

Program EVALPLOT  
(Version 2021-1)

by

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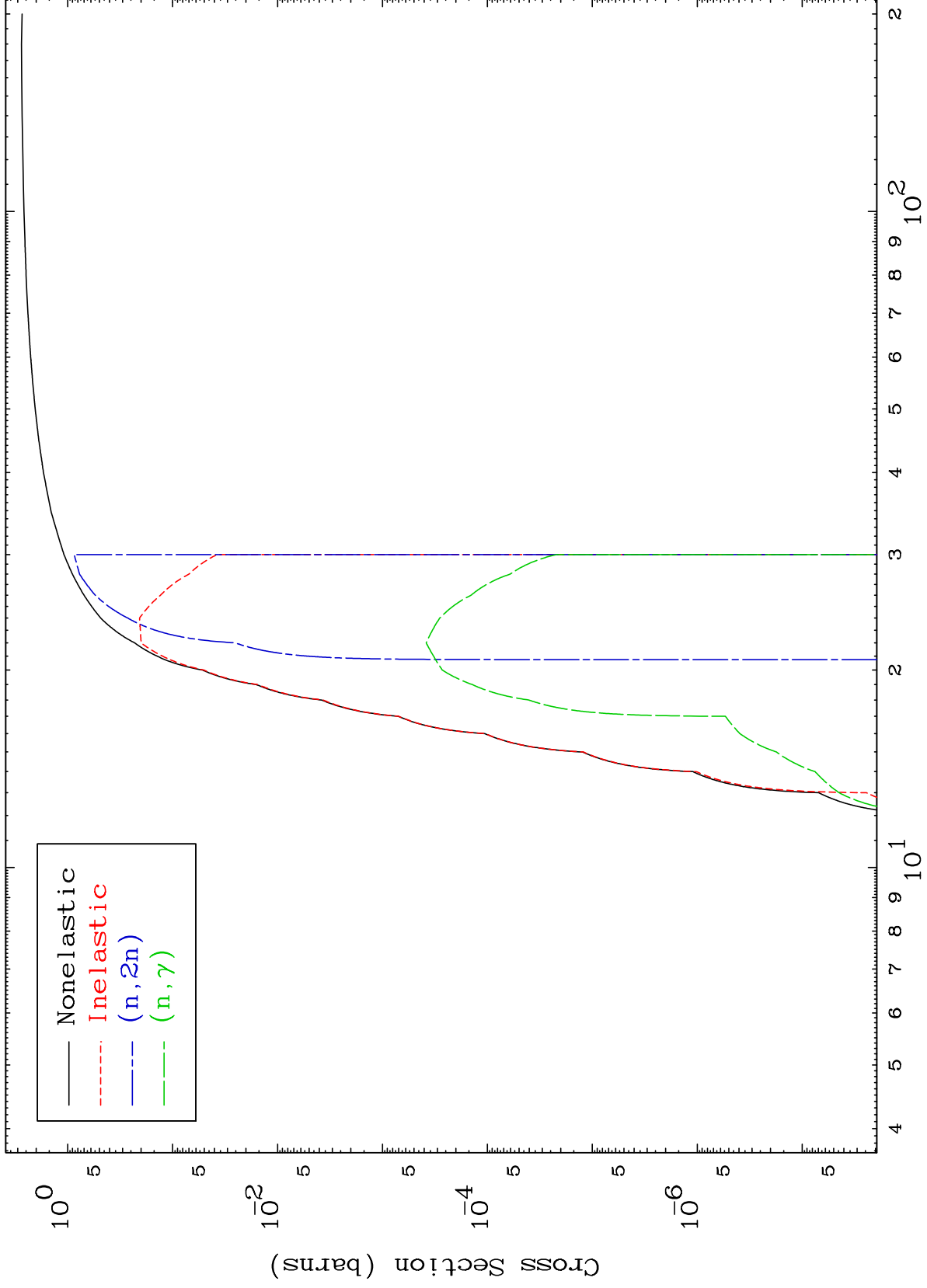
Press Mouse Button to Start

MAT 8111

$\alpha$  Major

0 Kelvin Cross Sections

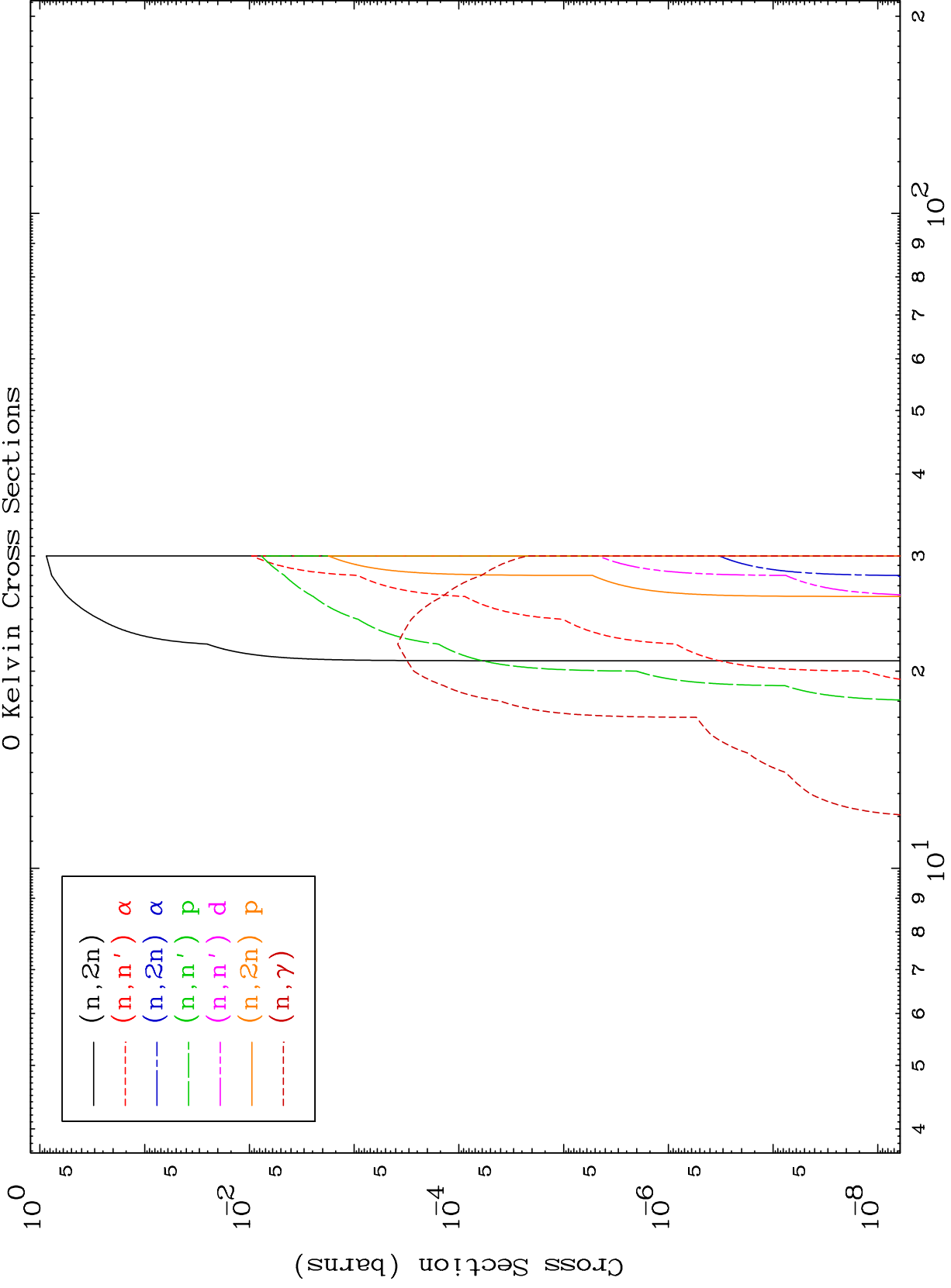
81-TI-198m



MAT 81111

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

81-Tl-198m



2

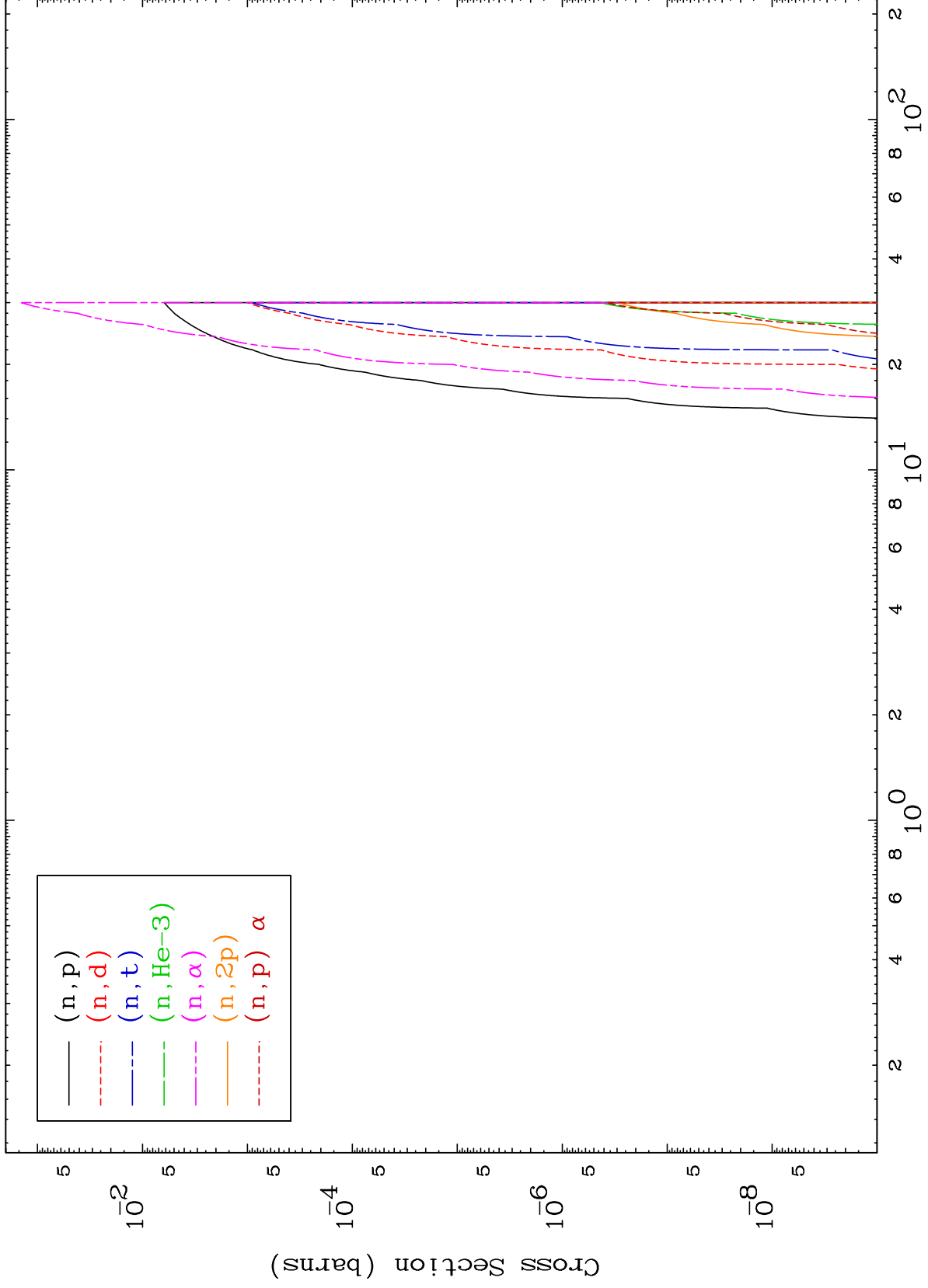
Incident Energy (MeV)

81-Tl-198m

MAT 8111

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

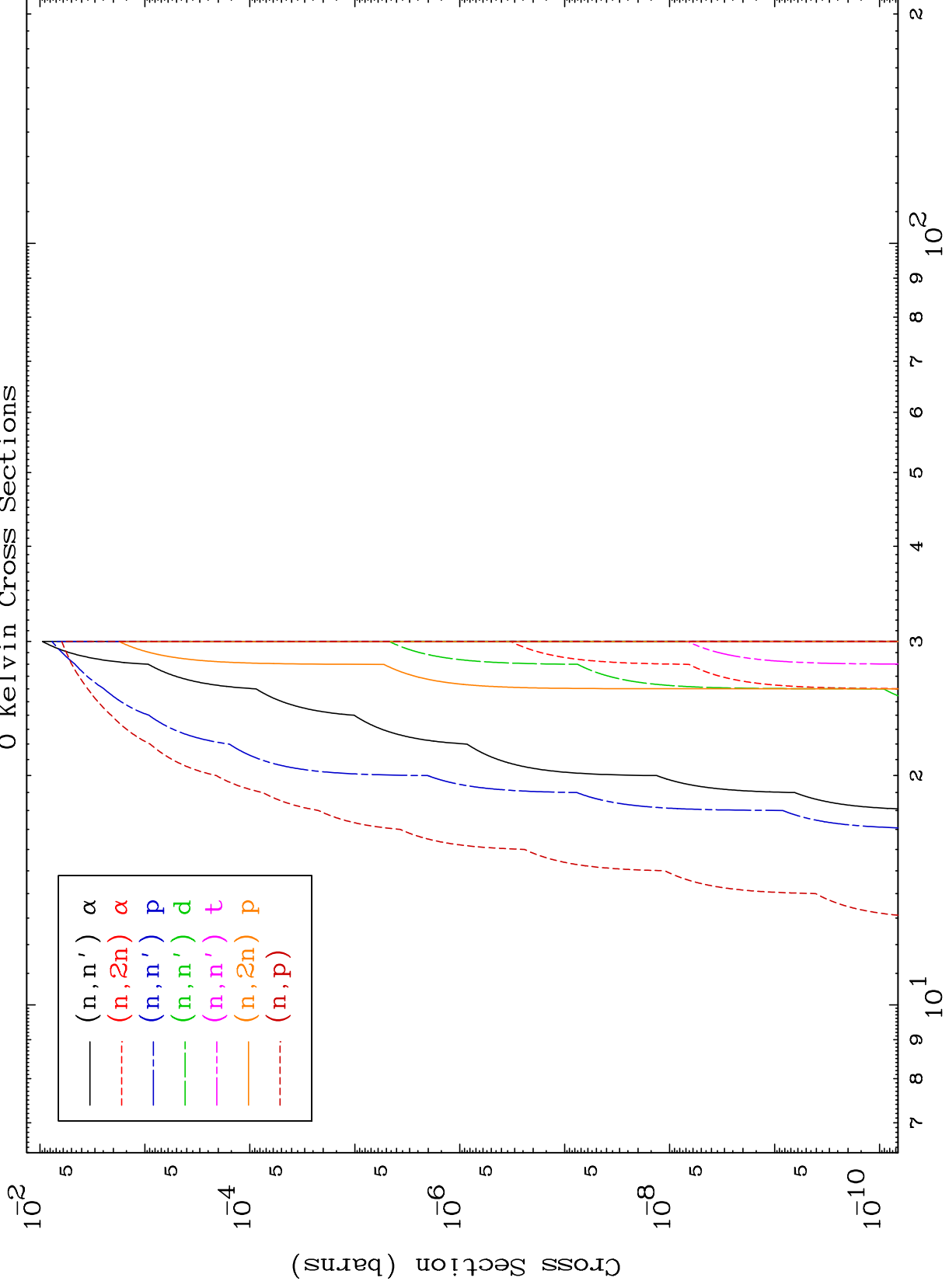
81-Tl-198m



MAT 81111

$\alpha$  Charged Particle  
0 Kelvin Cross Sections

81-Tl-198m



4

Incident Energy (MeV)

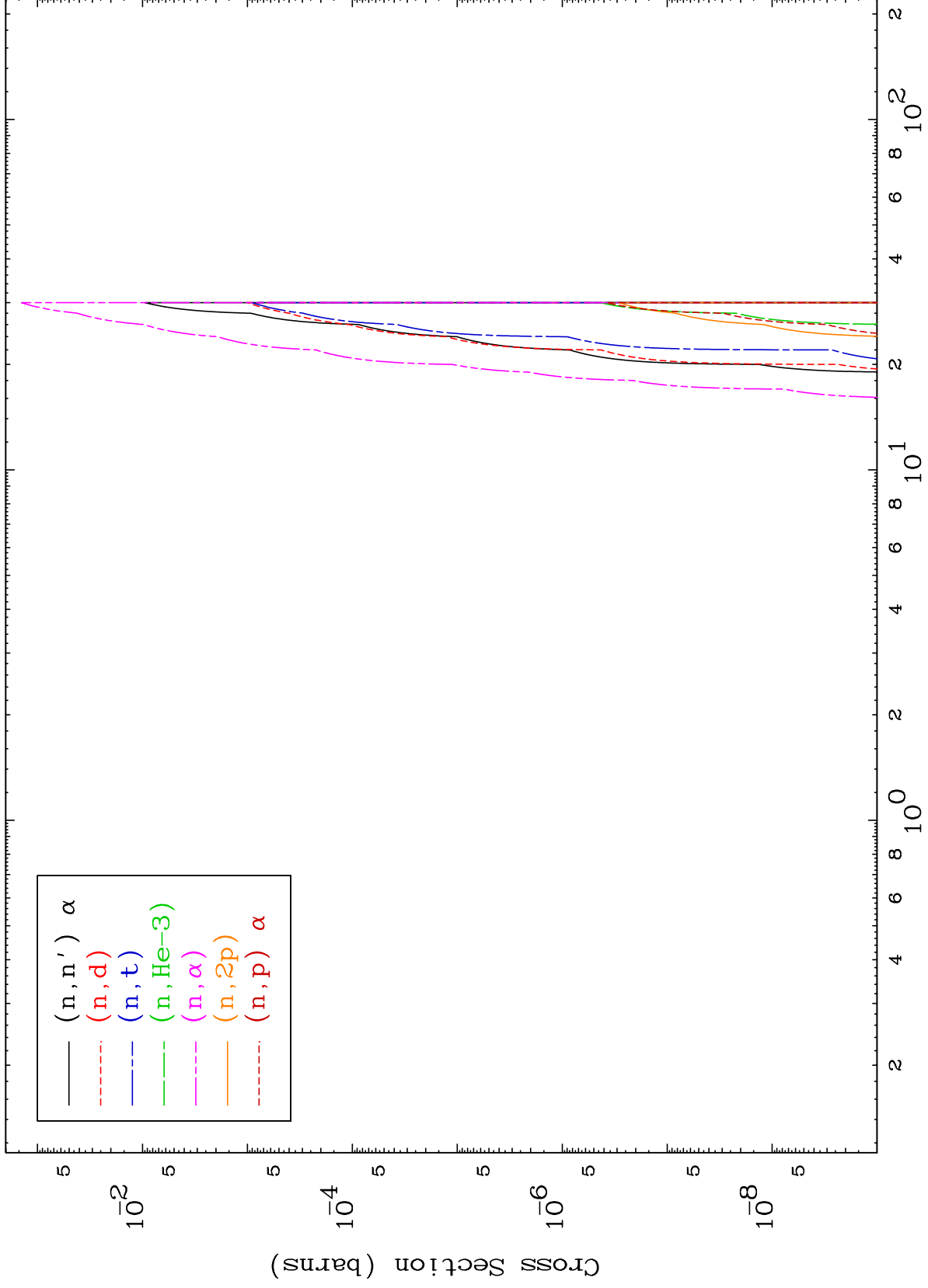
81-Tl-198m

MAT 81111

$\alpha$  Charged Particle

81-Tl-198m

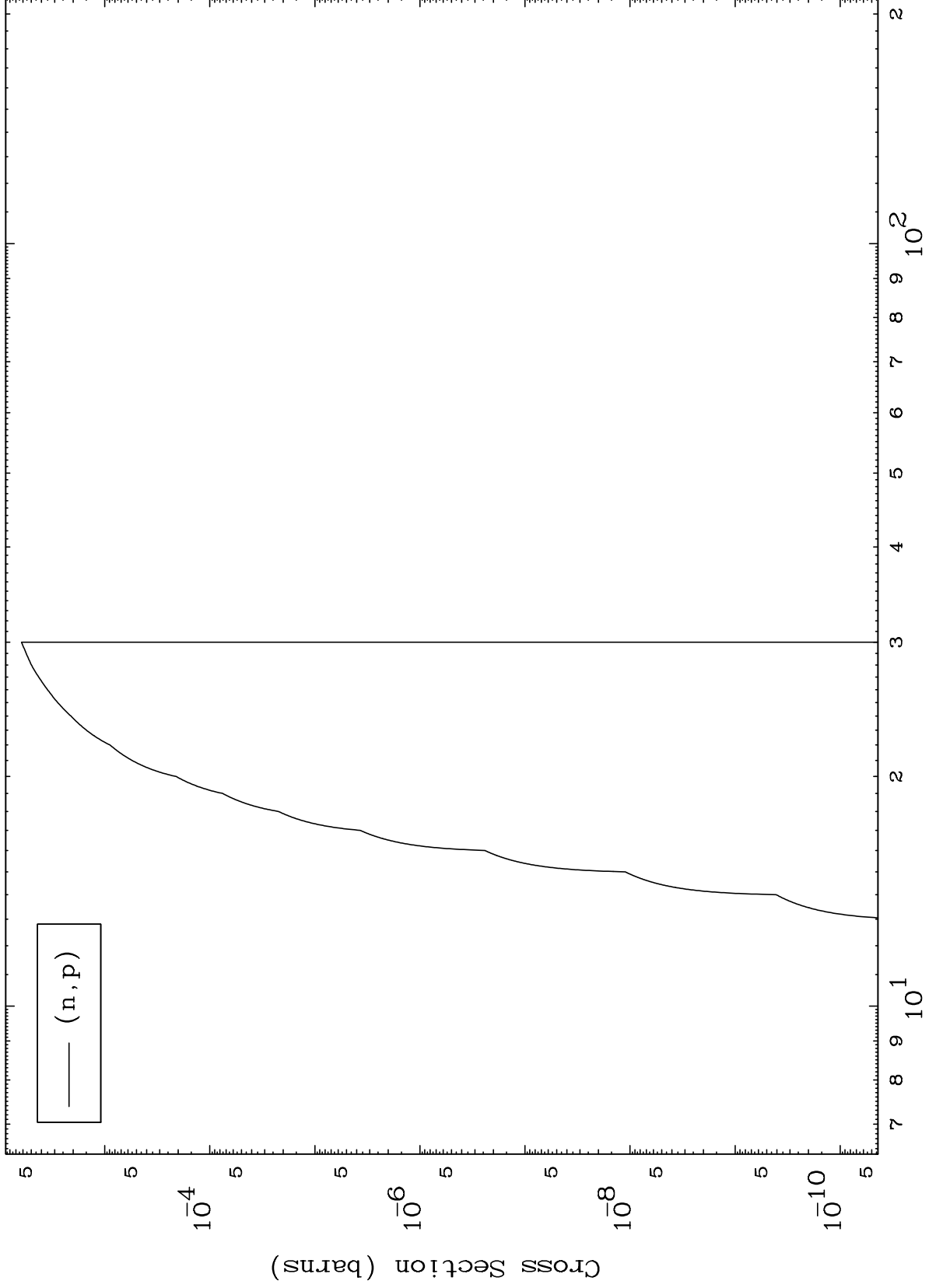
0 Kelvin Cross Sections



MAT 81111

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

81-Tl-198m



6

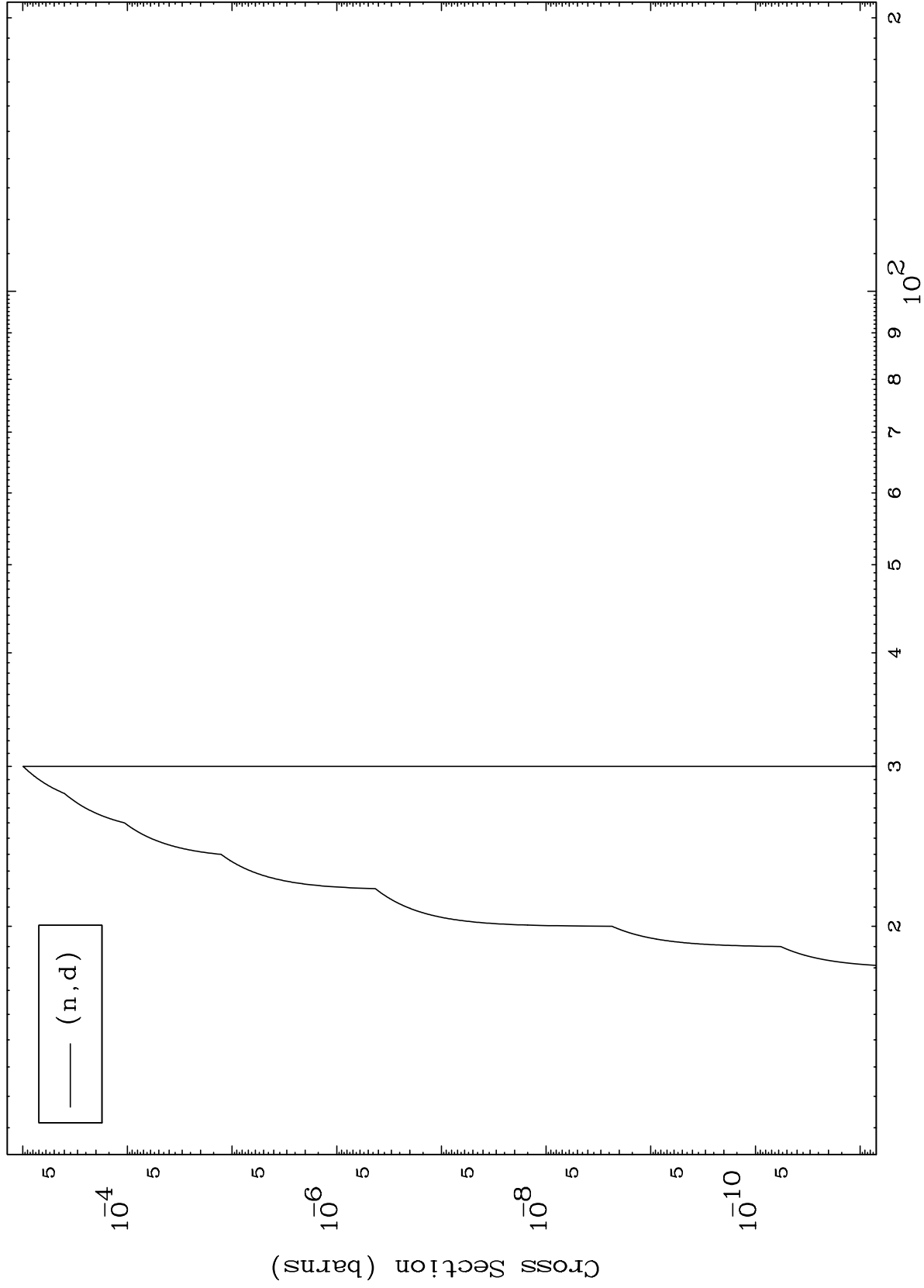
Incident Energy (MeV)

81-Tl-198m

MAT 81111

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

81-Tl-198m



7

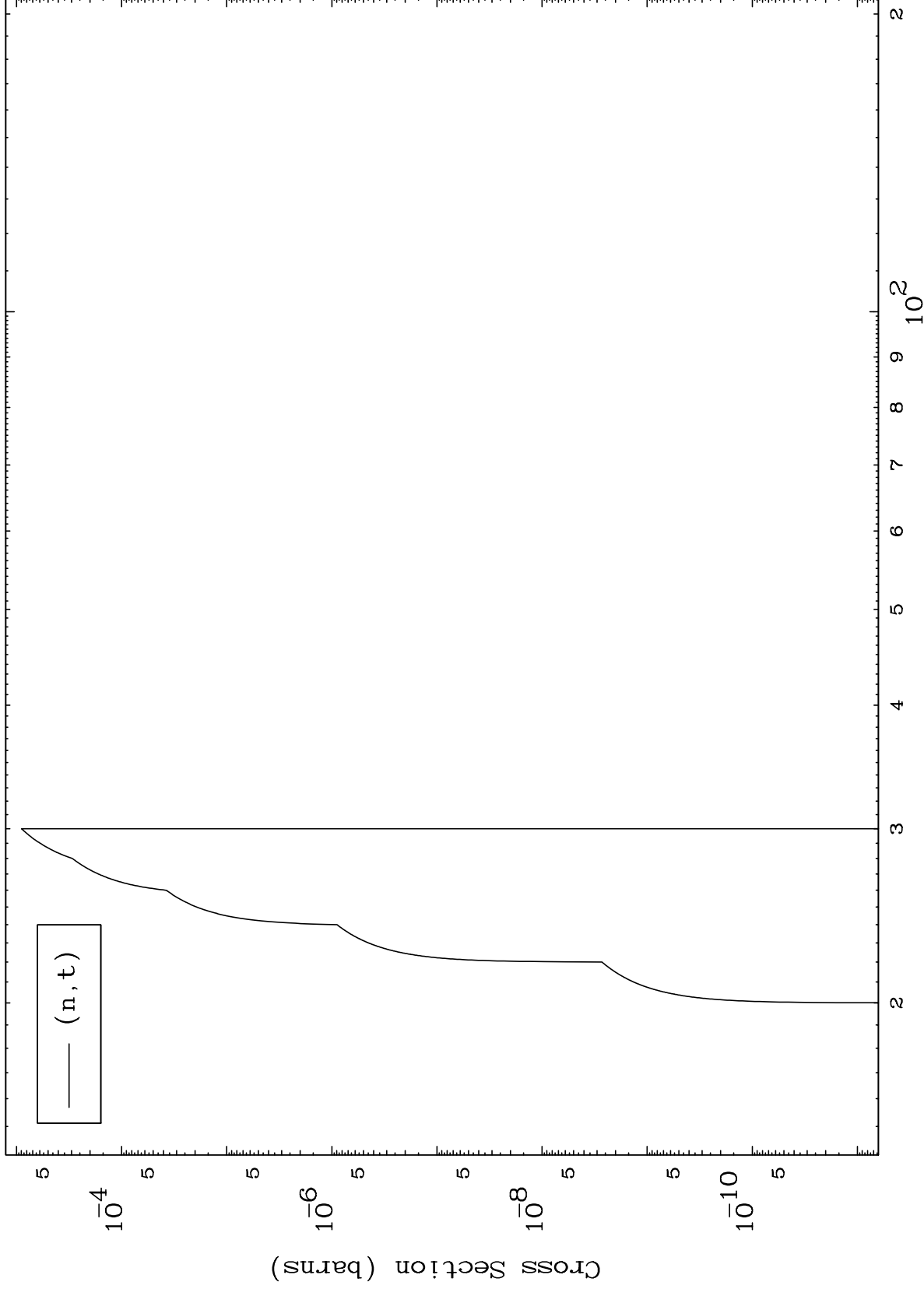
Incident Energy (MeV)

81-Tl-198m

MAT 81111

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

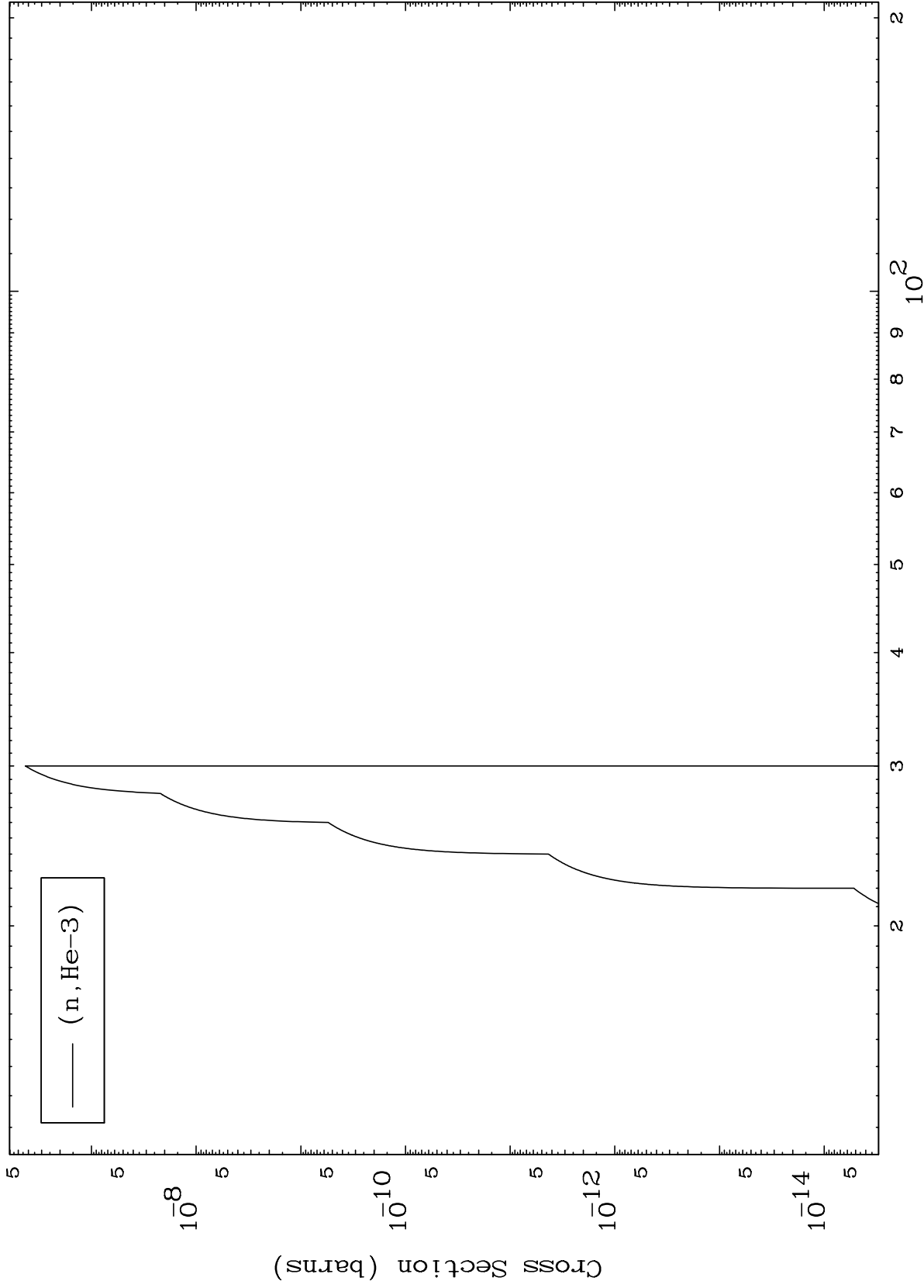
81-TI-198m



8

Incident Energy (MeV)

81-TI-198m

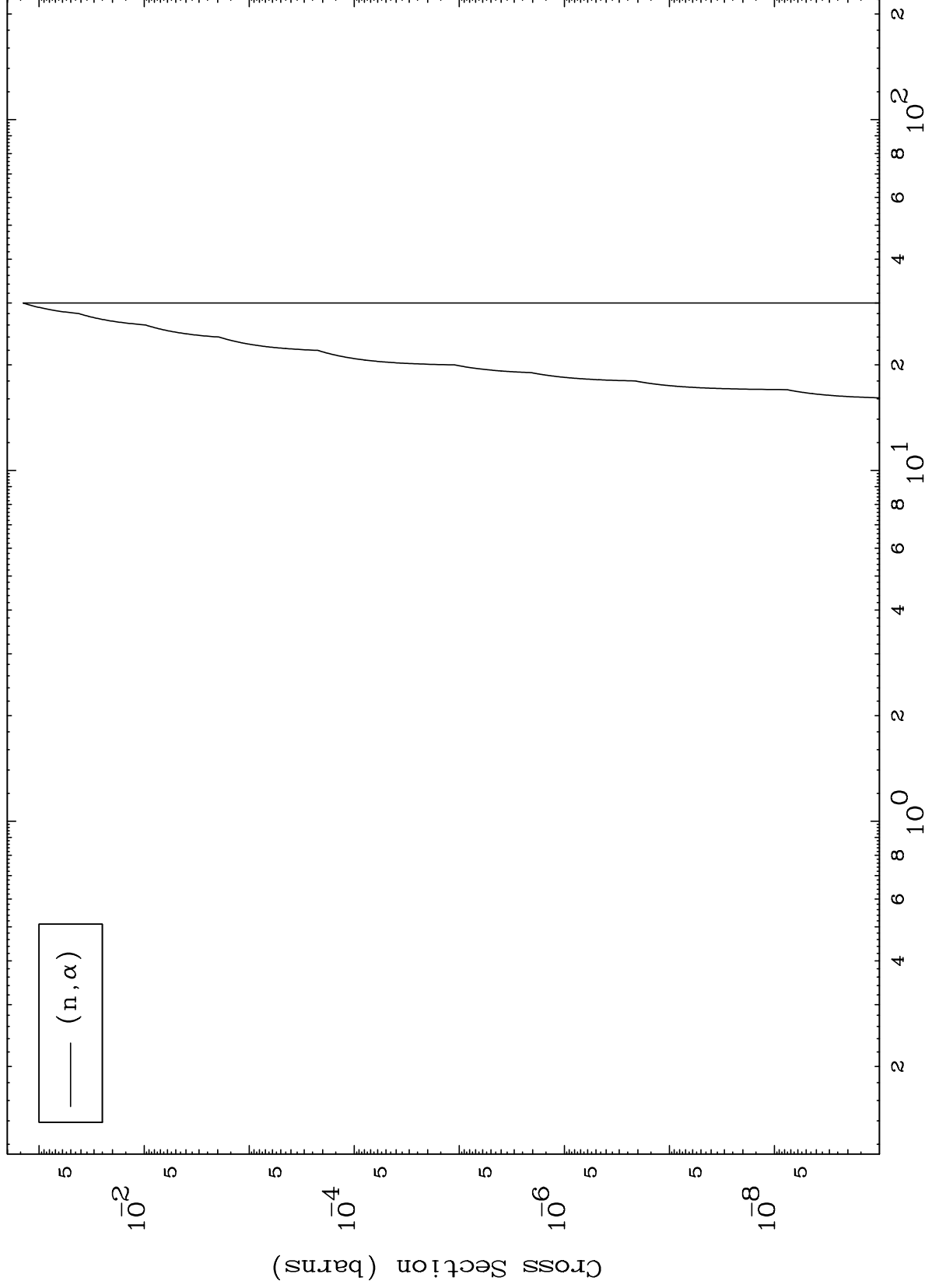


MAT 81111

( $\alpha, \alpha$ ) Levels

81-Tl-198m

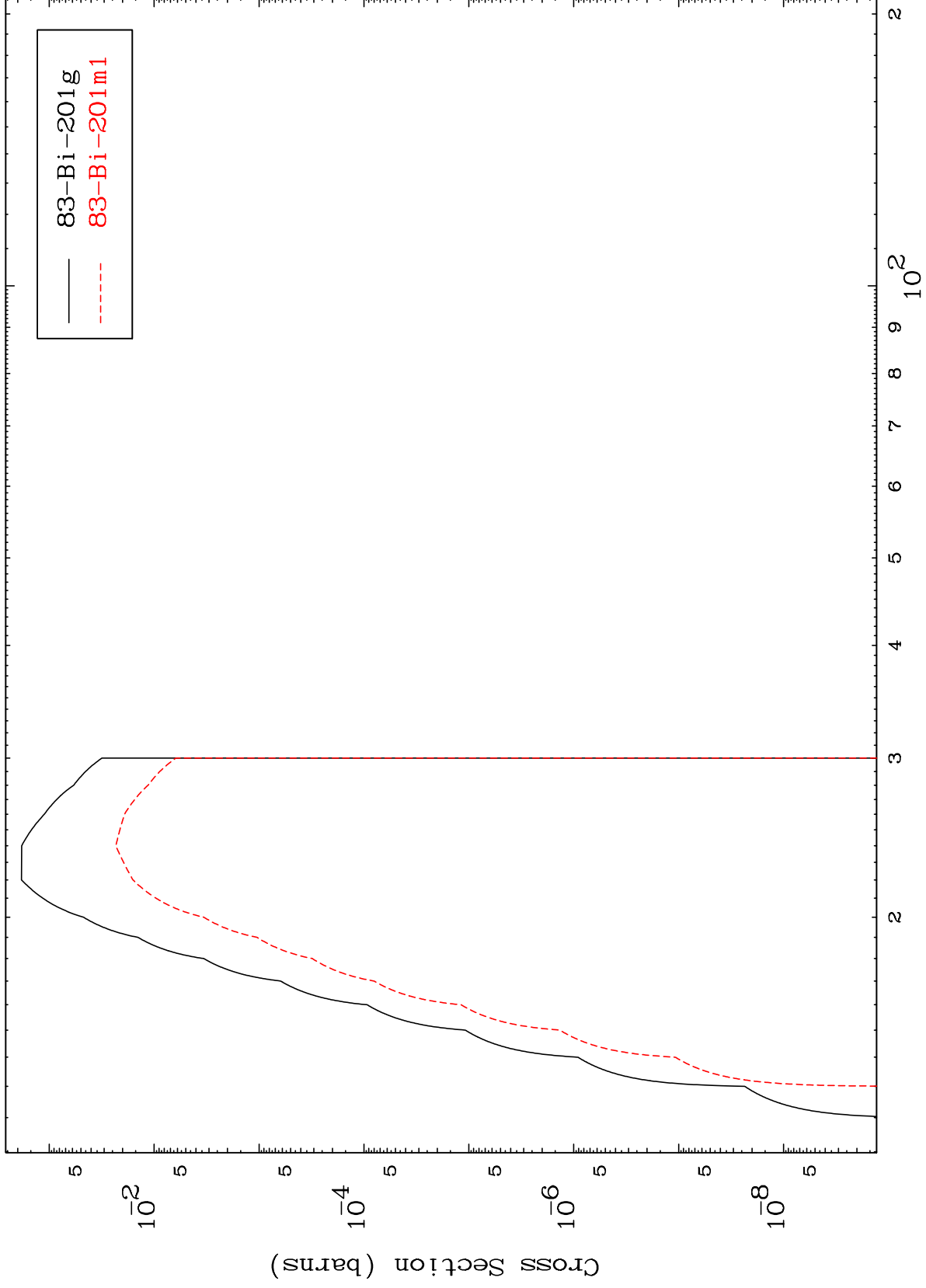
0 Kelvin Cross Sections



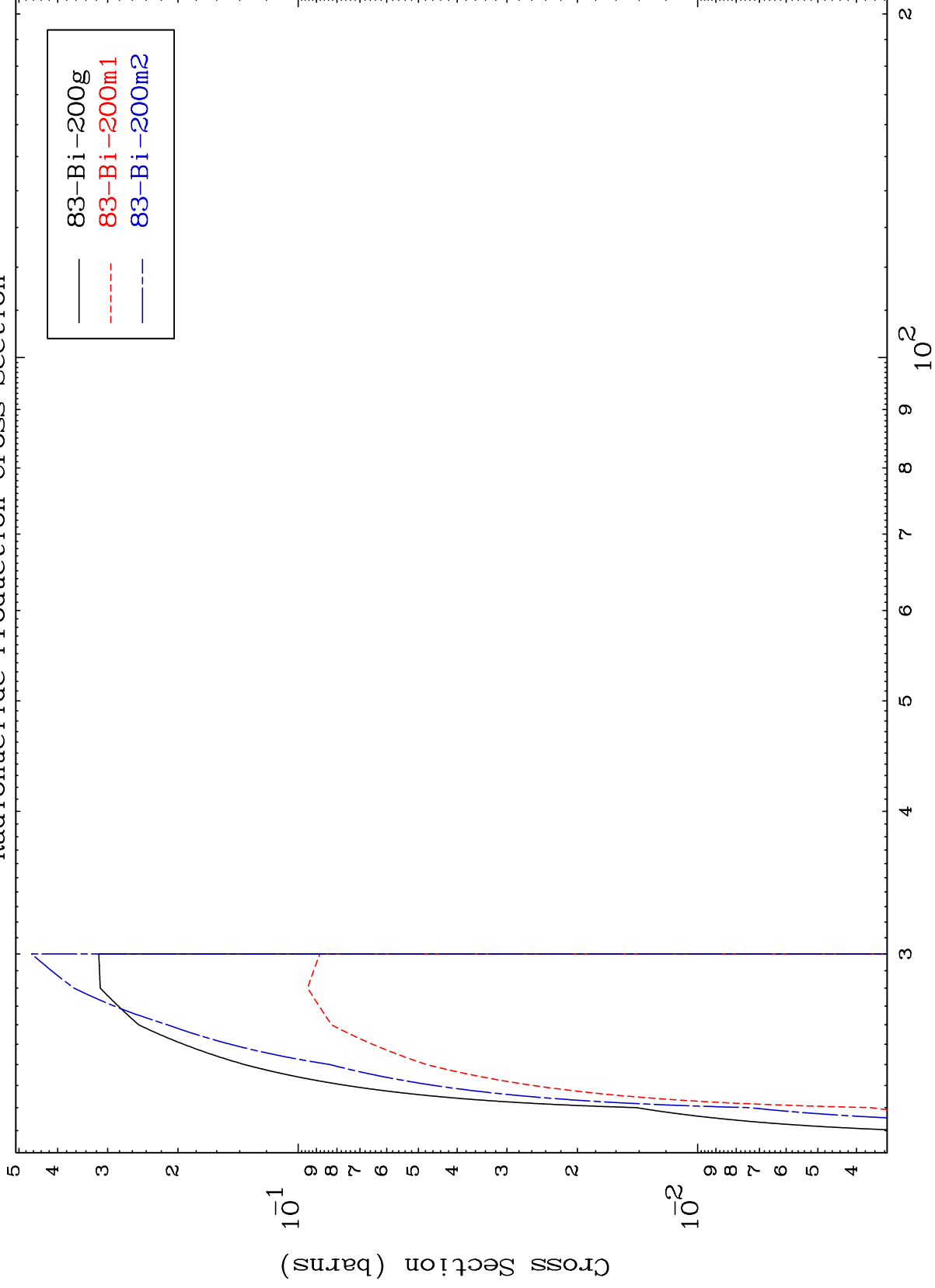
10

Incident Energy (MeV)

81-Tl-198m



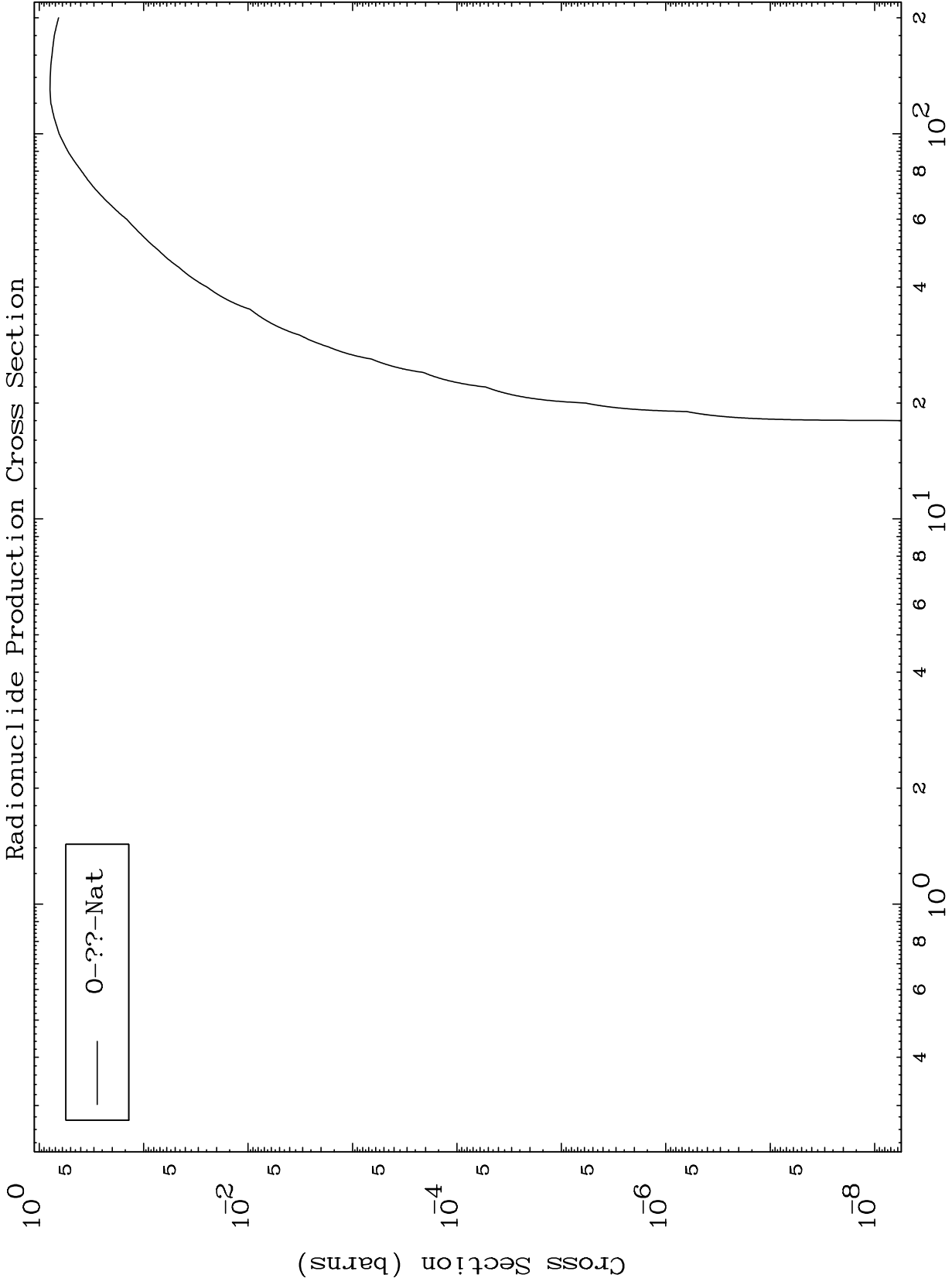
Radionuclide Production Cross Section



MAT 81111

Fission

81-Tl-198m

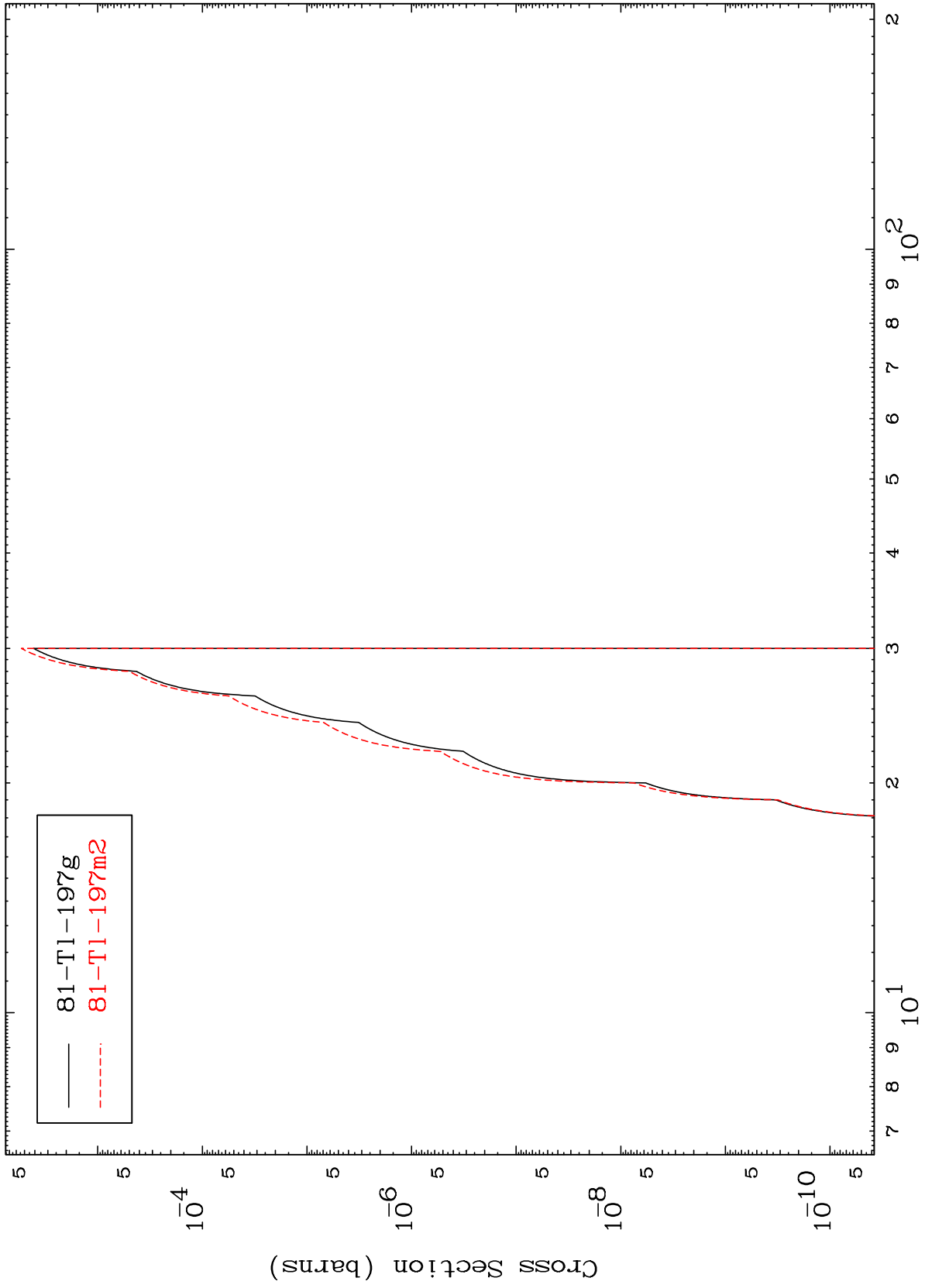


MAT 81111

$(n, n') \alpha$

81-Tl-198m

Radionuclide Production Cross Section



14

Incident Energy (MeV)

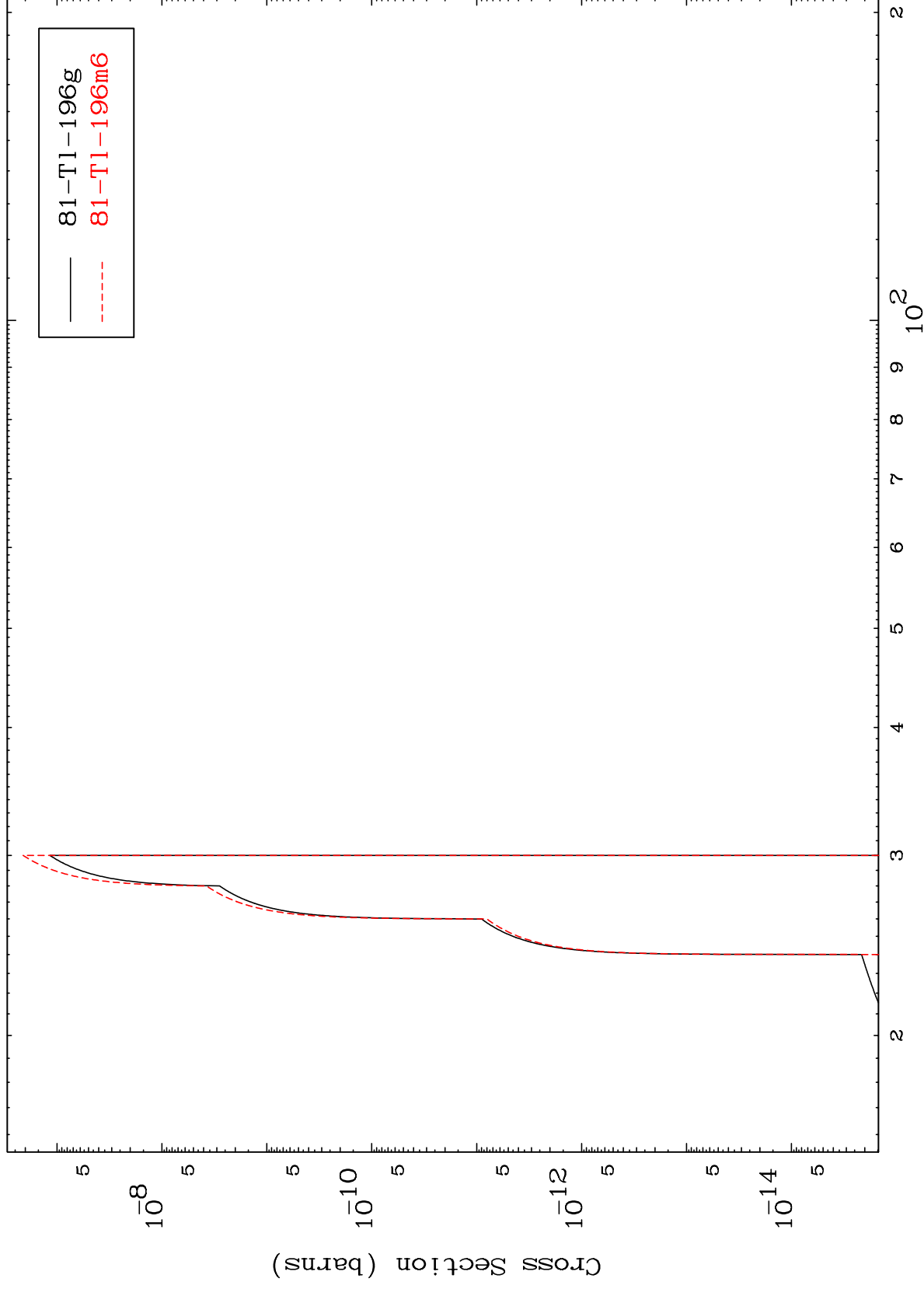
81-Tl-198m

MAT 81111

$(n,2n) \alpha$

81-Tl-198m

Radionuclide Production Cross Section



15

Incident Energy (MeV)

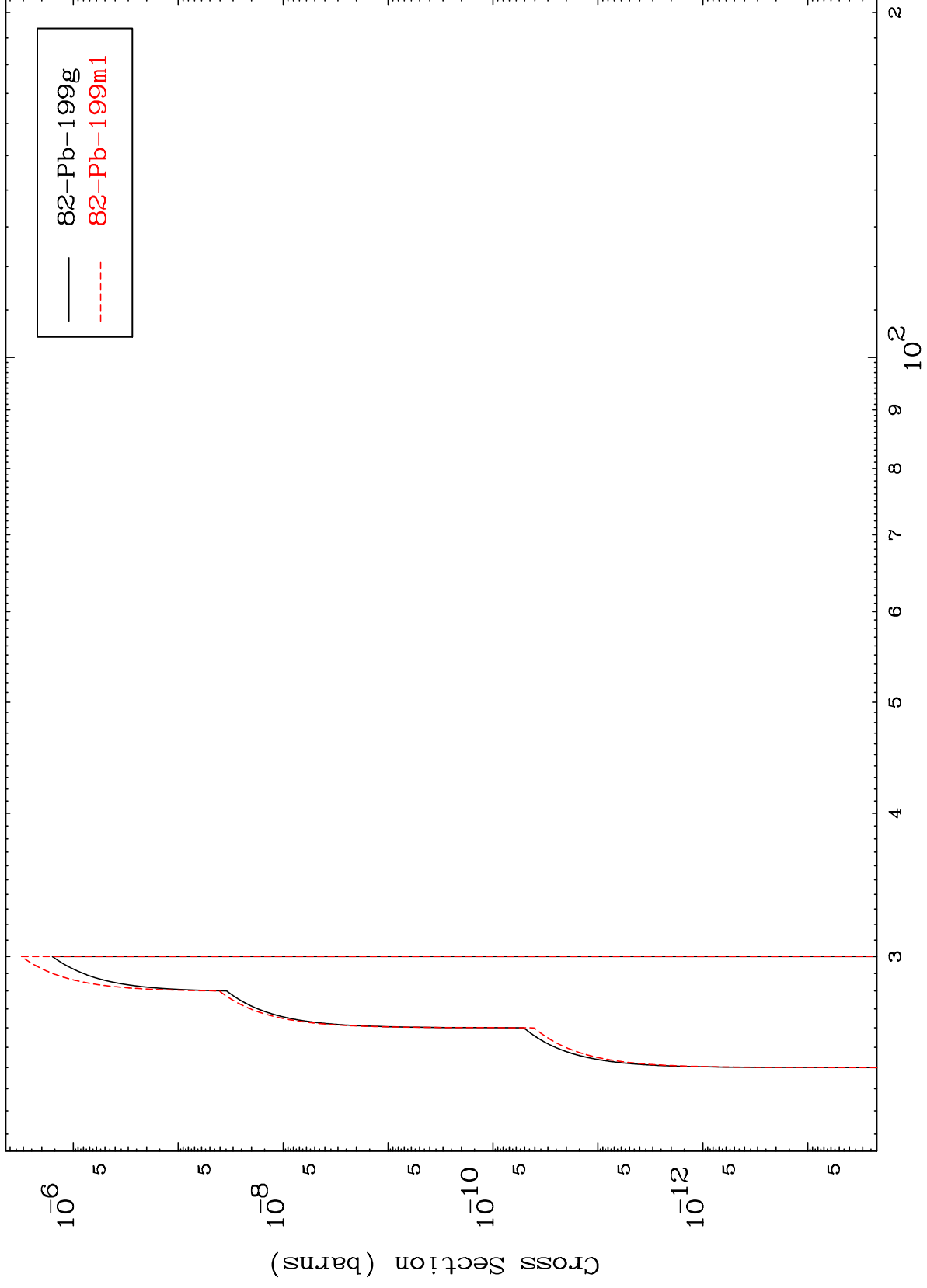
81-Tl-198m

MAT 81111

(n,n') d

81-Tl-198m

Radionuclide Production Cross Section



16

Incident Energy (MeV)

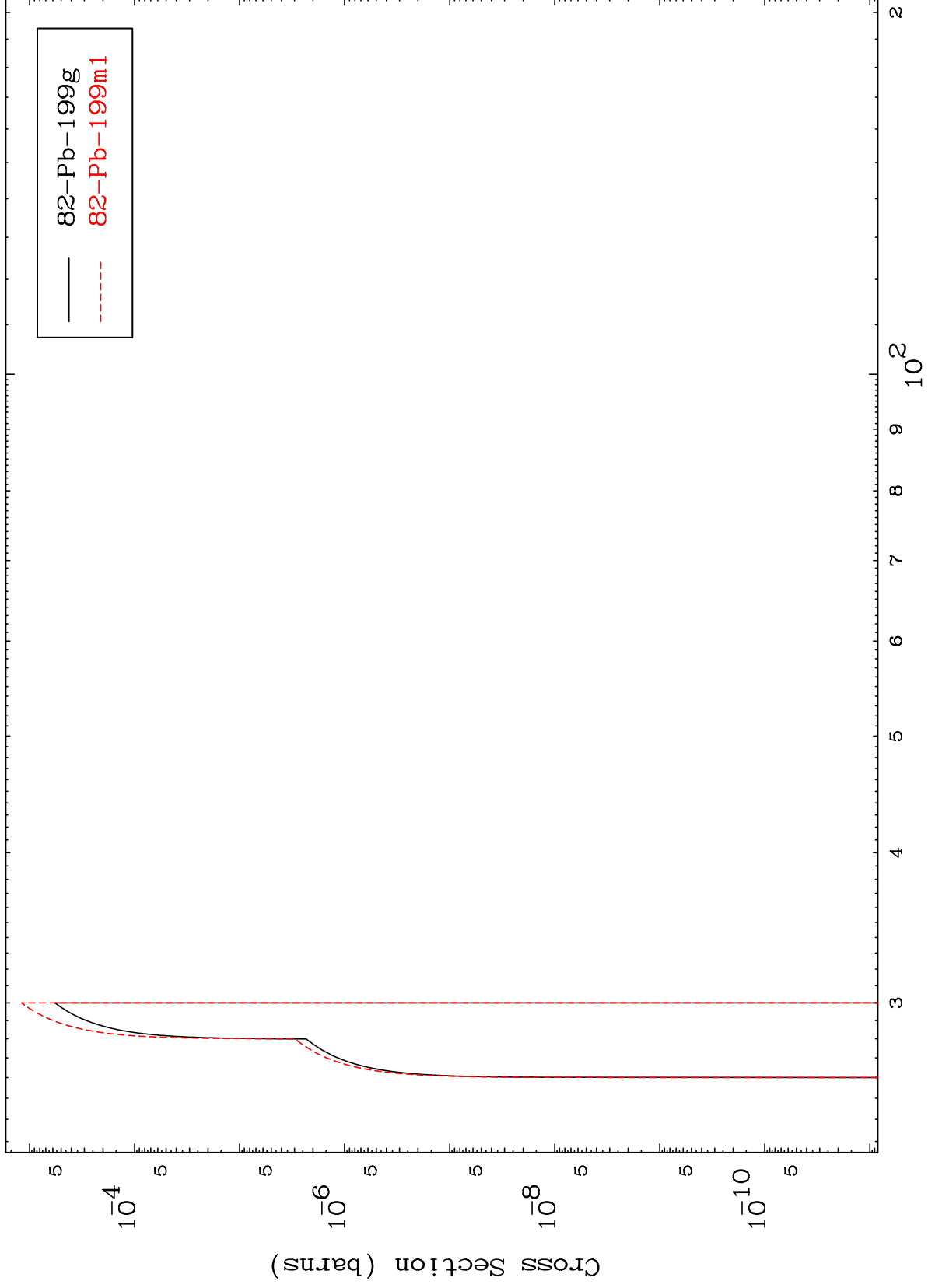
81-Tl-198m

MAT 8111

(n,2n) p

81-TI-198m

Radionuclide Production Cross Section



17

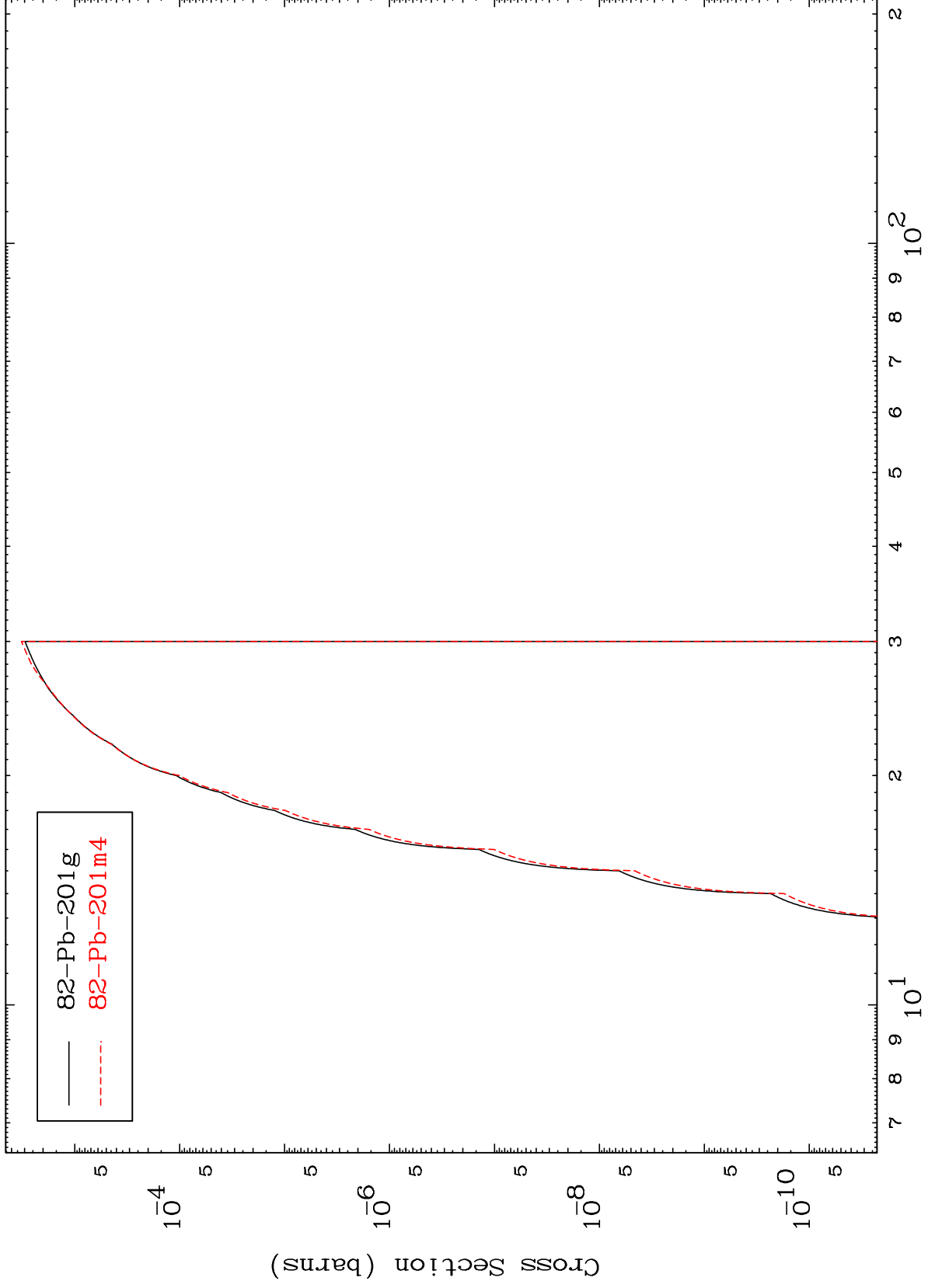
Incident Energy (MeV)

81-TI-198m

MAT 81111

81-TI-198m

(n,p)  
Radionuclide Production Cross Section



18

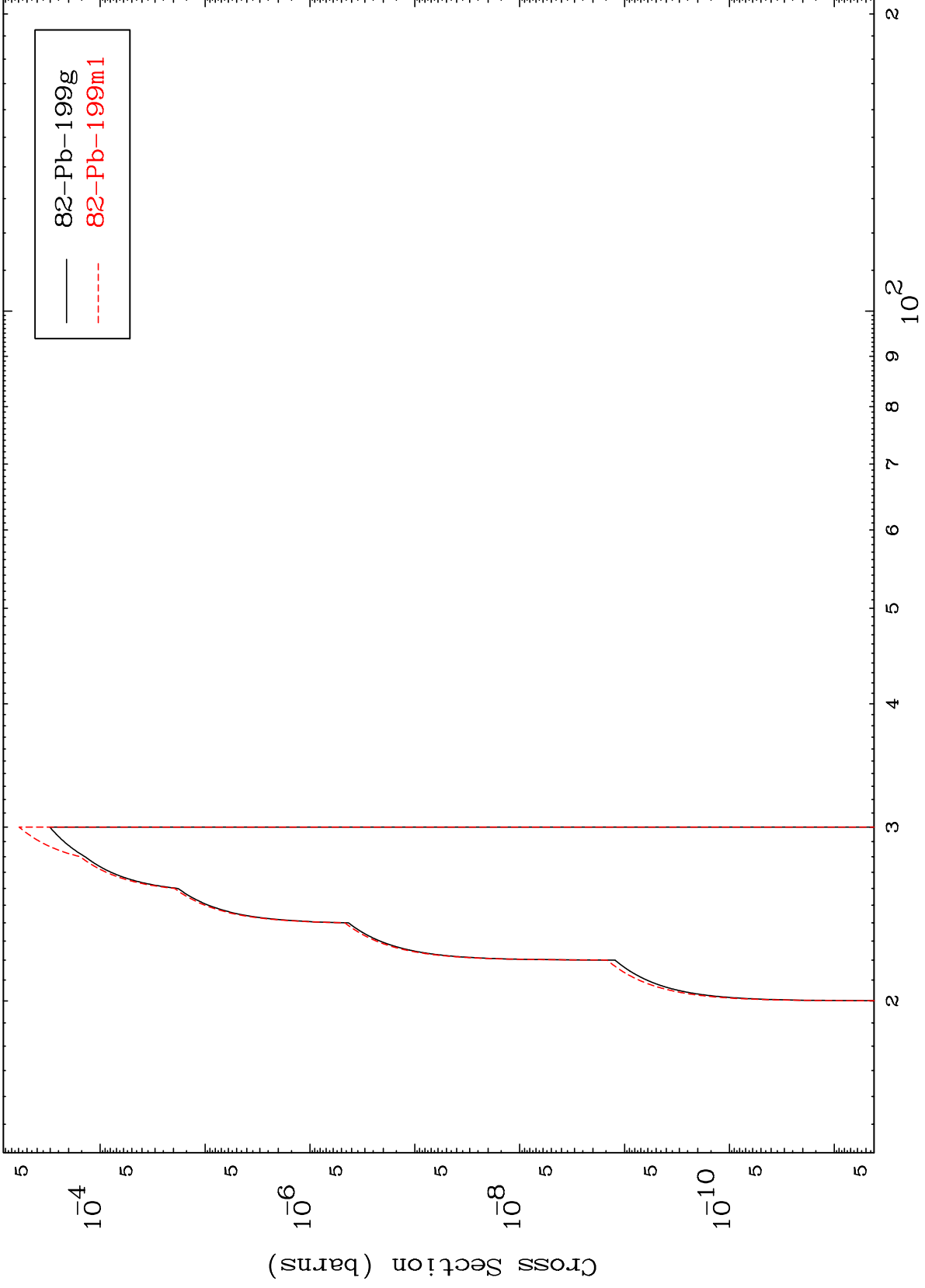
Incident Energy (MeV)

81-TI-198m

MAT 81111

81-Tl-198m

(n,t)  
Radionuclide Production Cross Section



19

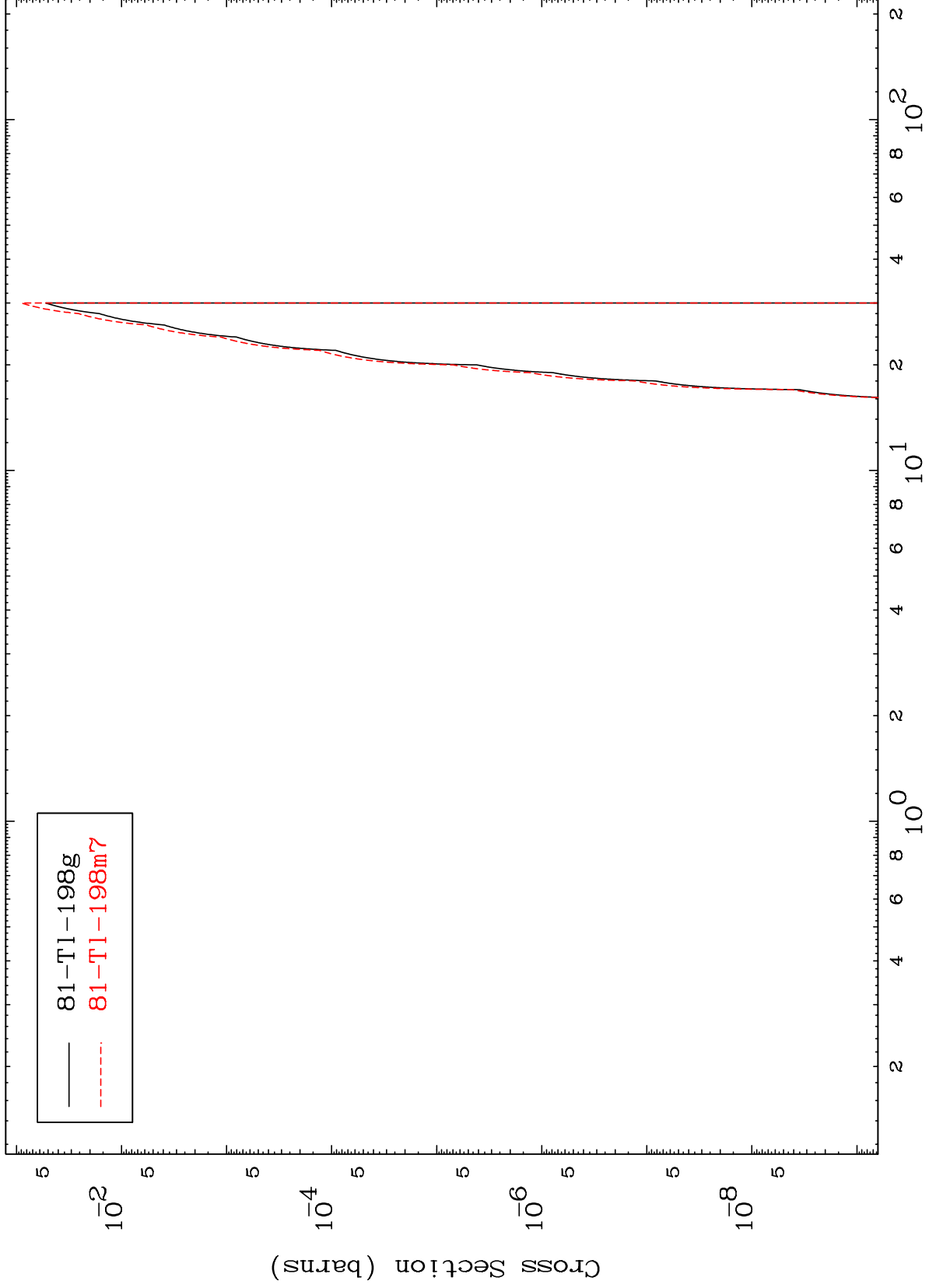
Incident Energy (MeV)

81-Tl-198m

MAT 81111

81-Tl-198m

Radionuclide Production Cross Section



20

81-Tl-198m

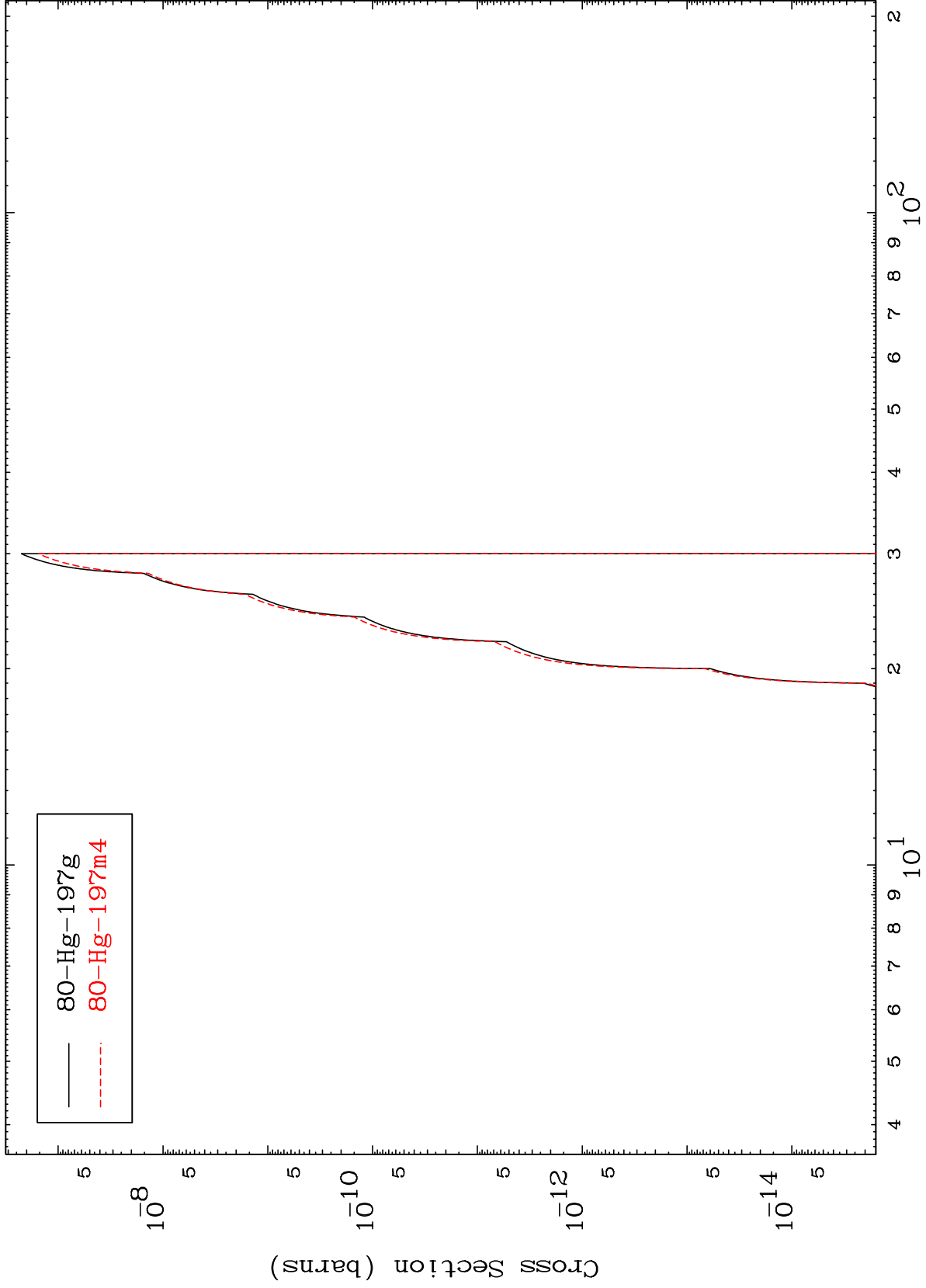
Incident Energy (MeV)

MAT 81111

(n,p)  $\alpha$

81-Tl-198m

Radionuclide Production Cross Section



21

Incident Energy (MeV)

81-Tl-198m