

Program EVALPLOT  
(Version 2021-1)

by

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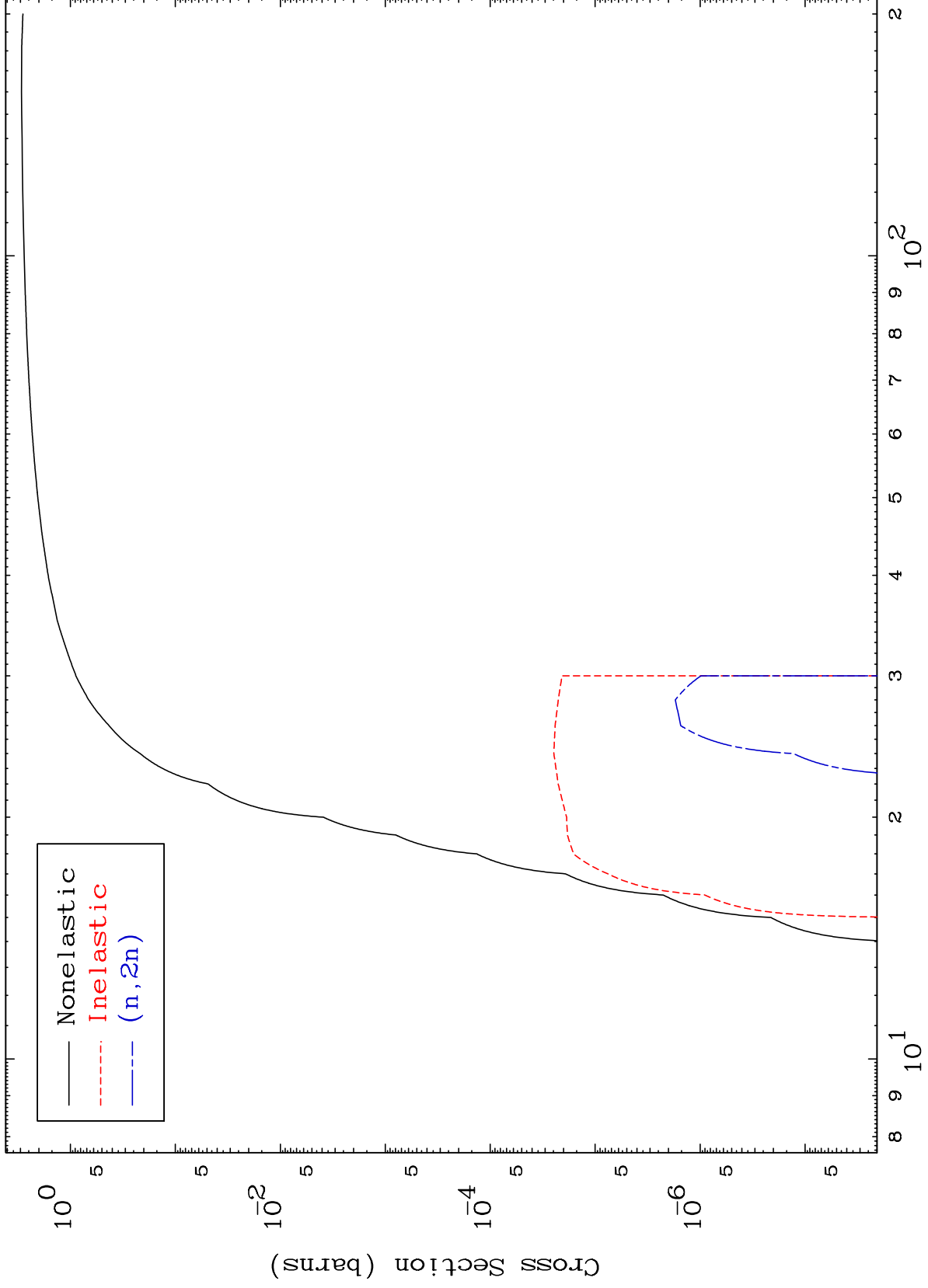
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9204

0 Kelvin Cross Sections

92-U -227



1

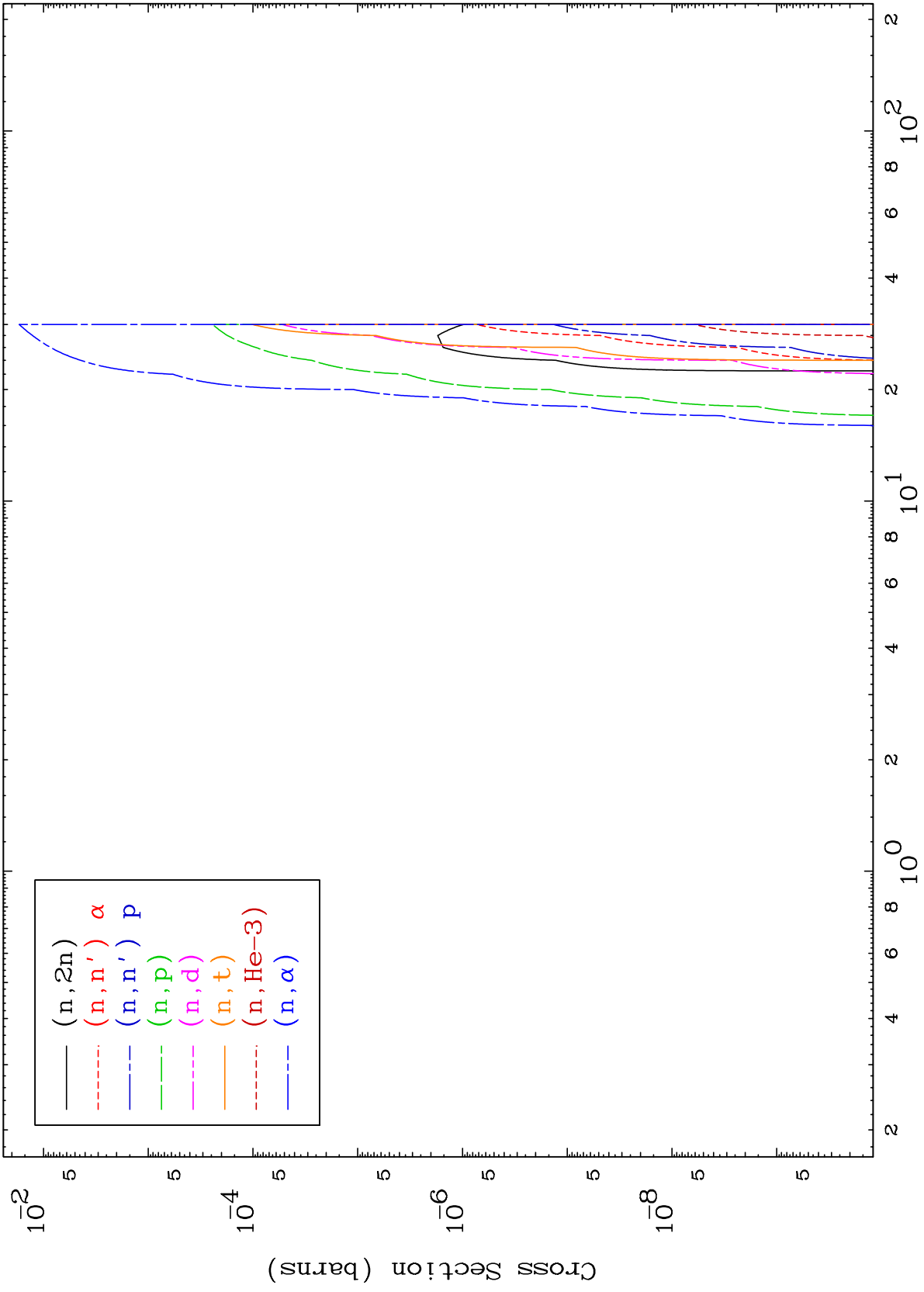
Incident Energy (MeV)

92-U -227

MAT 9204

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

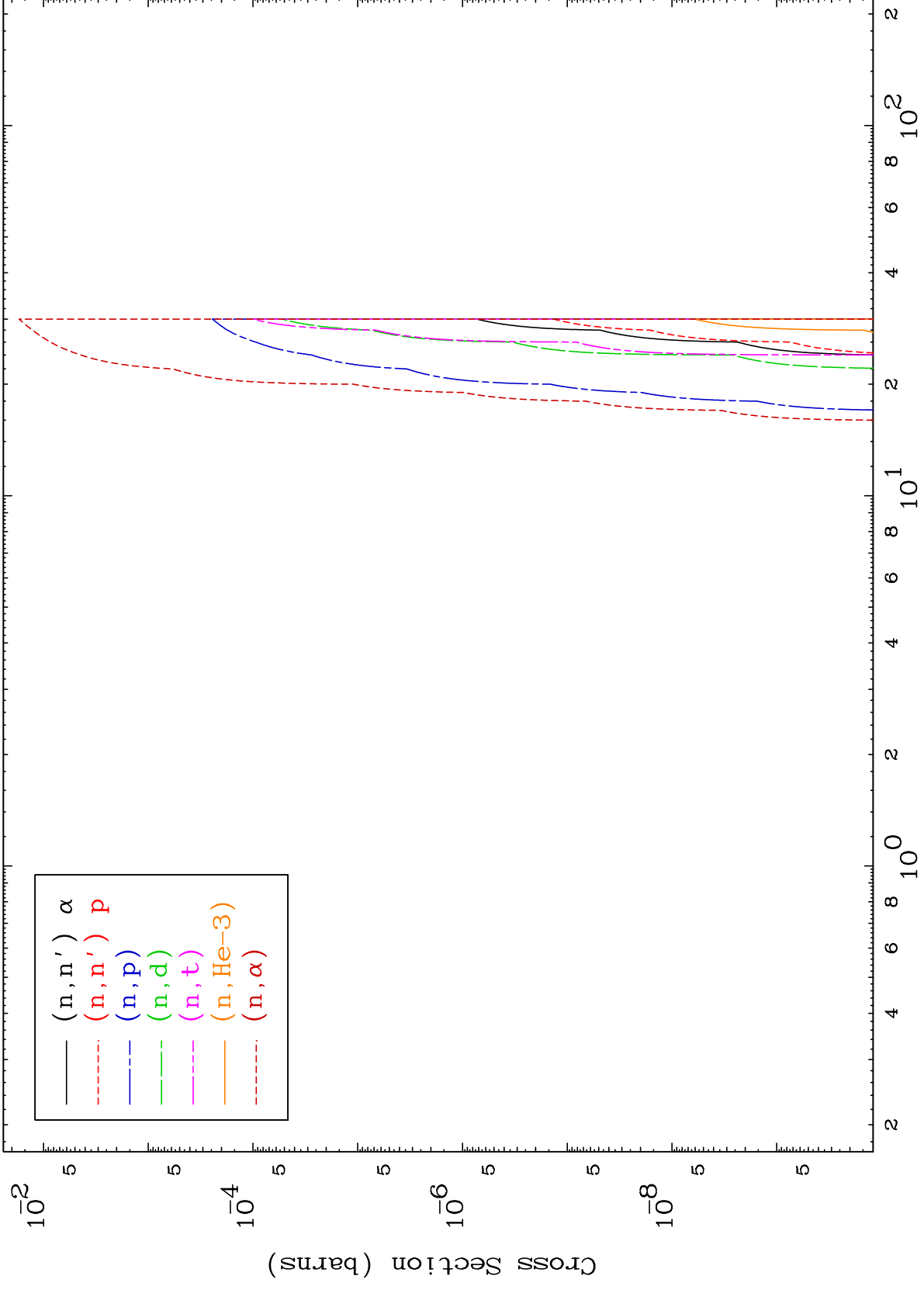
92-U -227



2

Incident Energy (MeV)

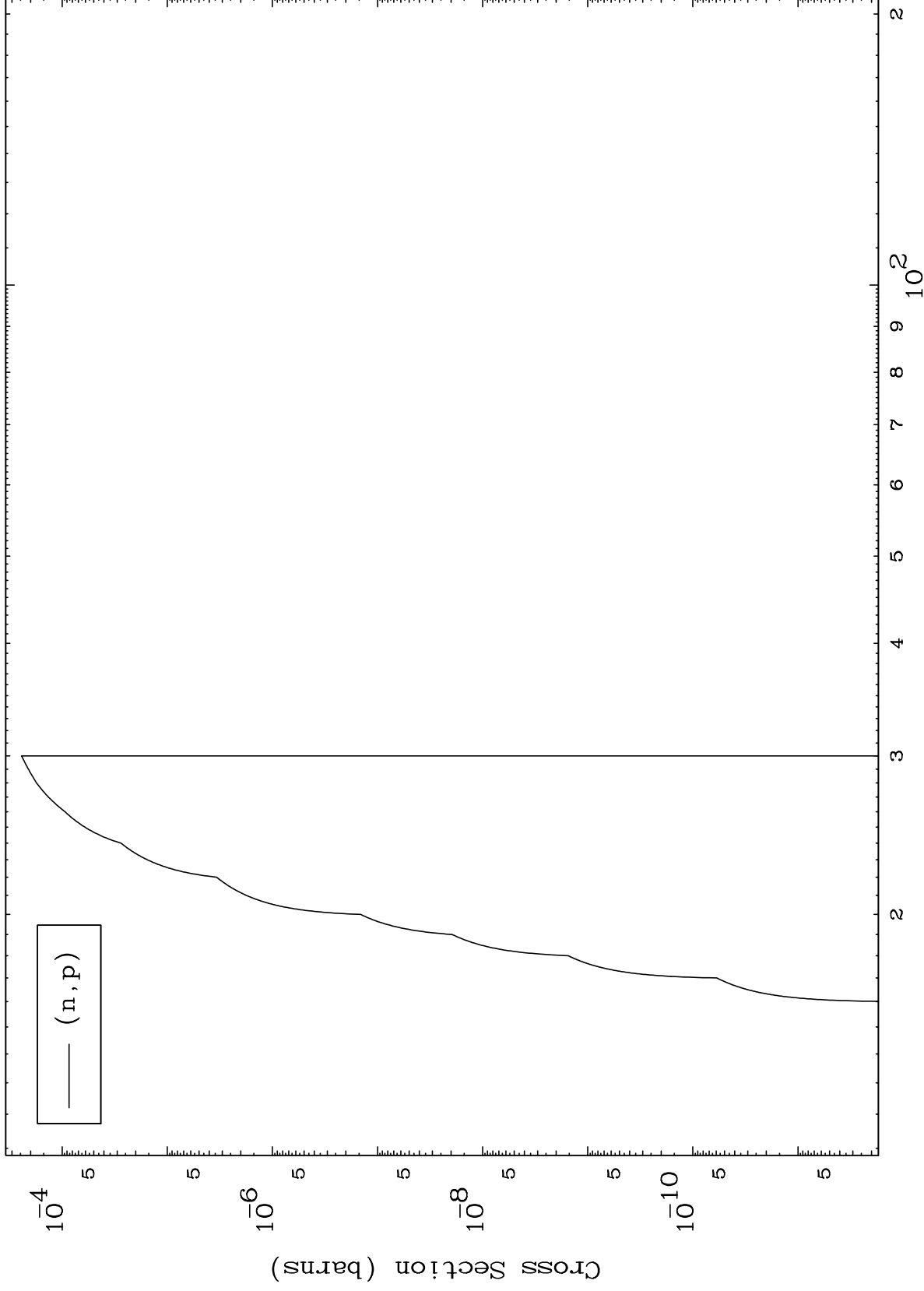
92-U -227



MAT 9204

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

92-U -227



Incident Energy (MeV)

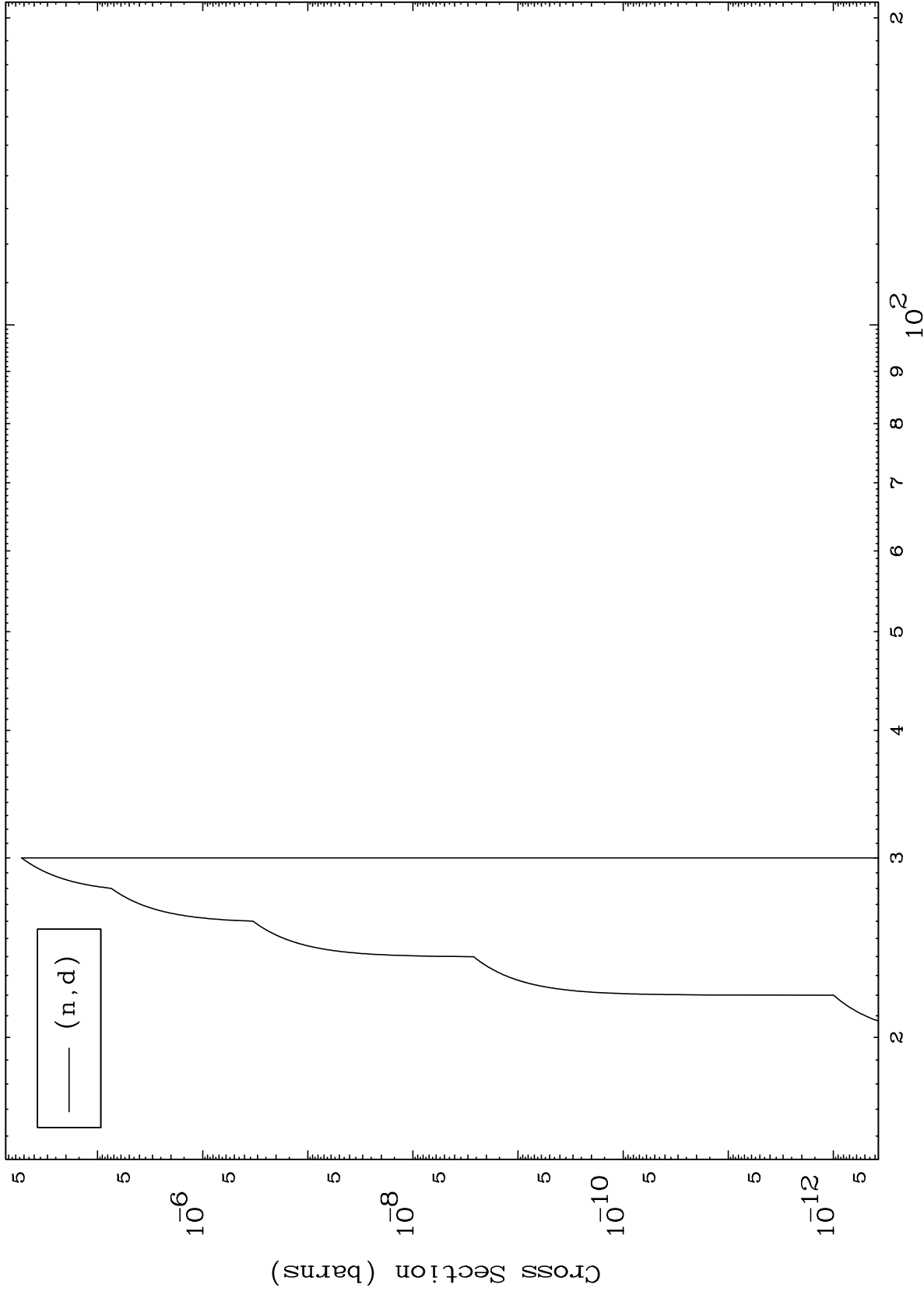
92-U -227

4

MAT 9204

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

92-U -227



5

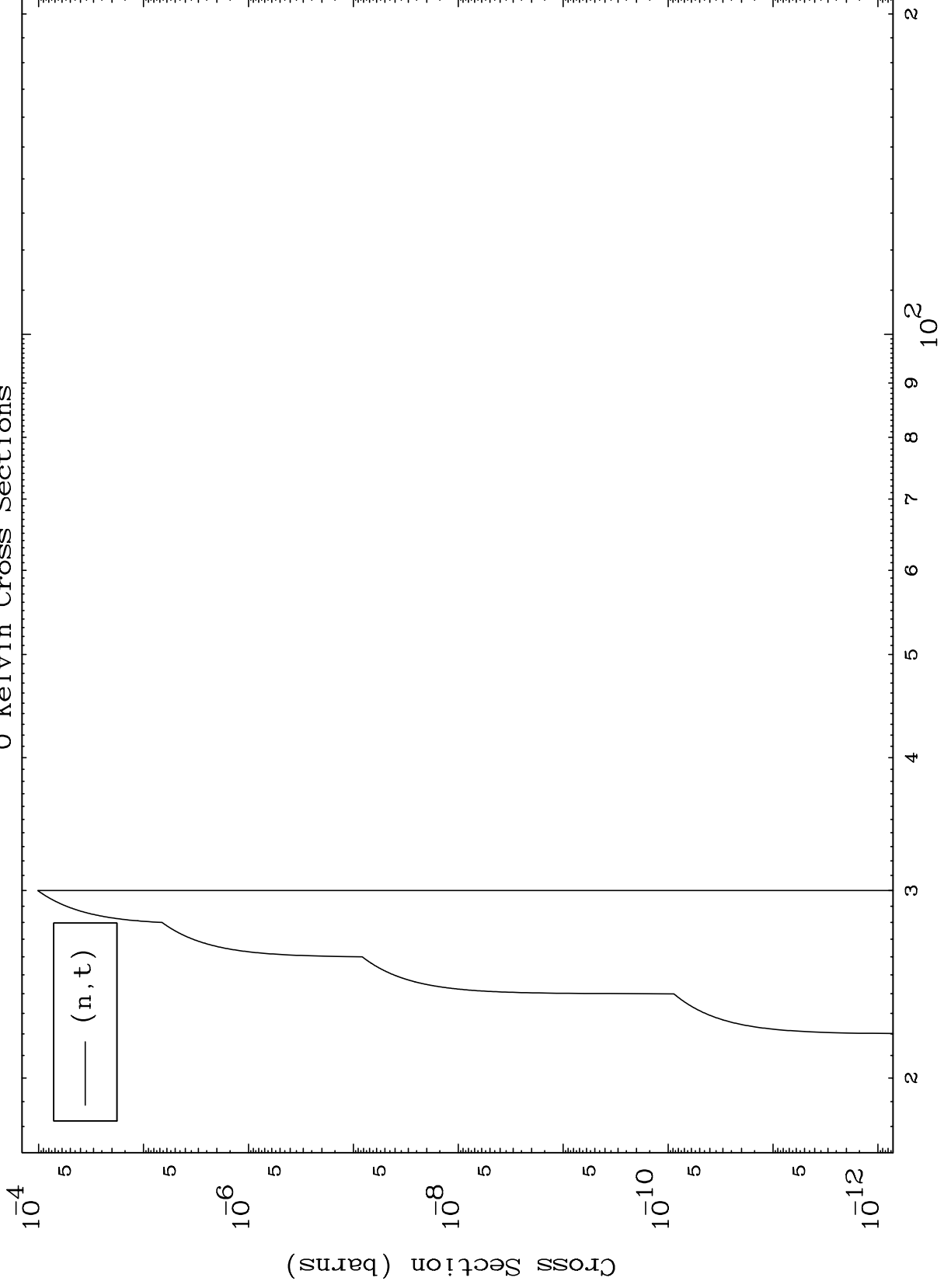
Incident Energy (MeV)

92-U -227

MAT 9204

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

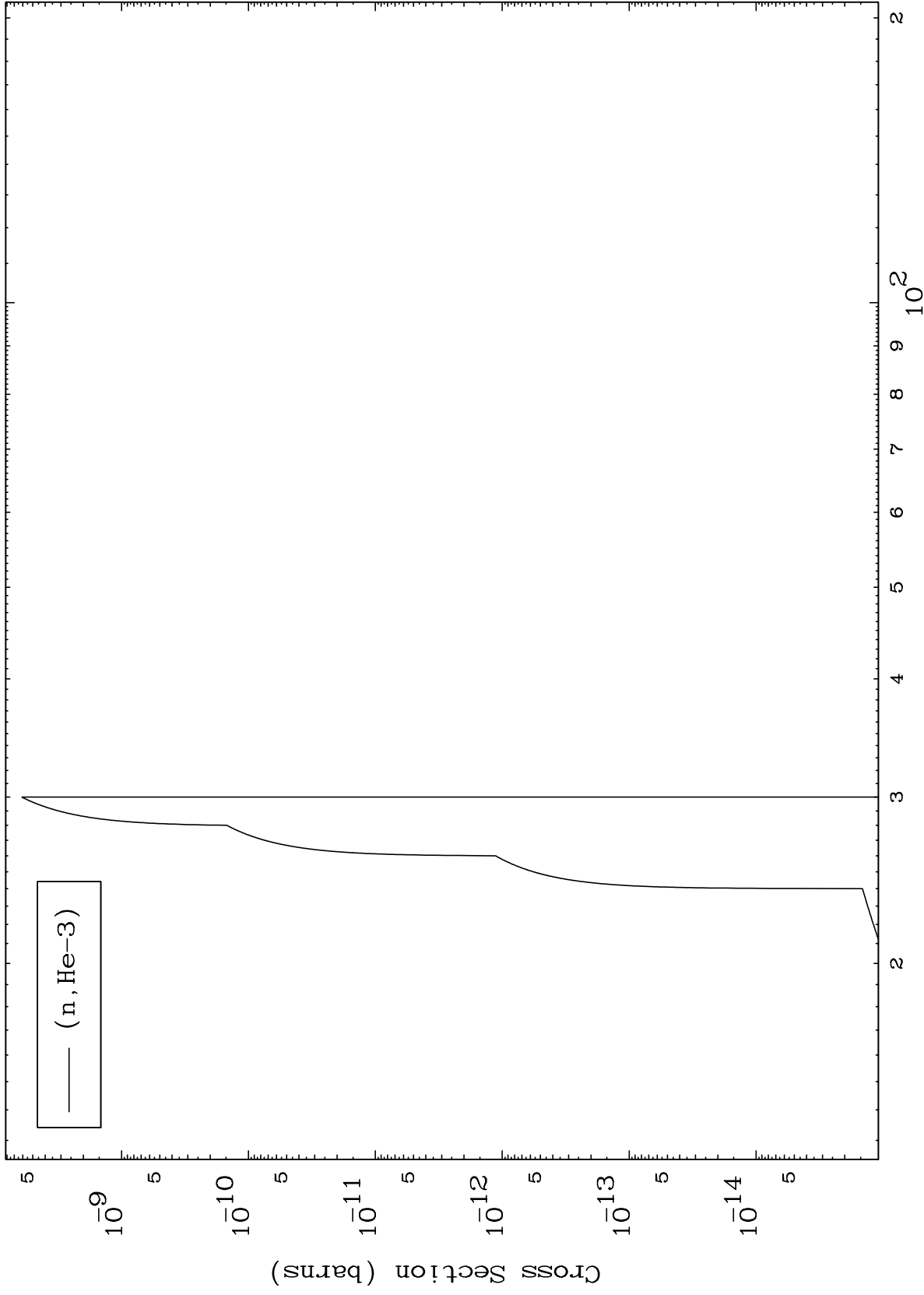
92-U -227



6

Incident Energy (MeV)

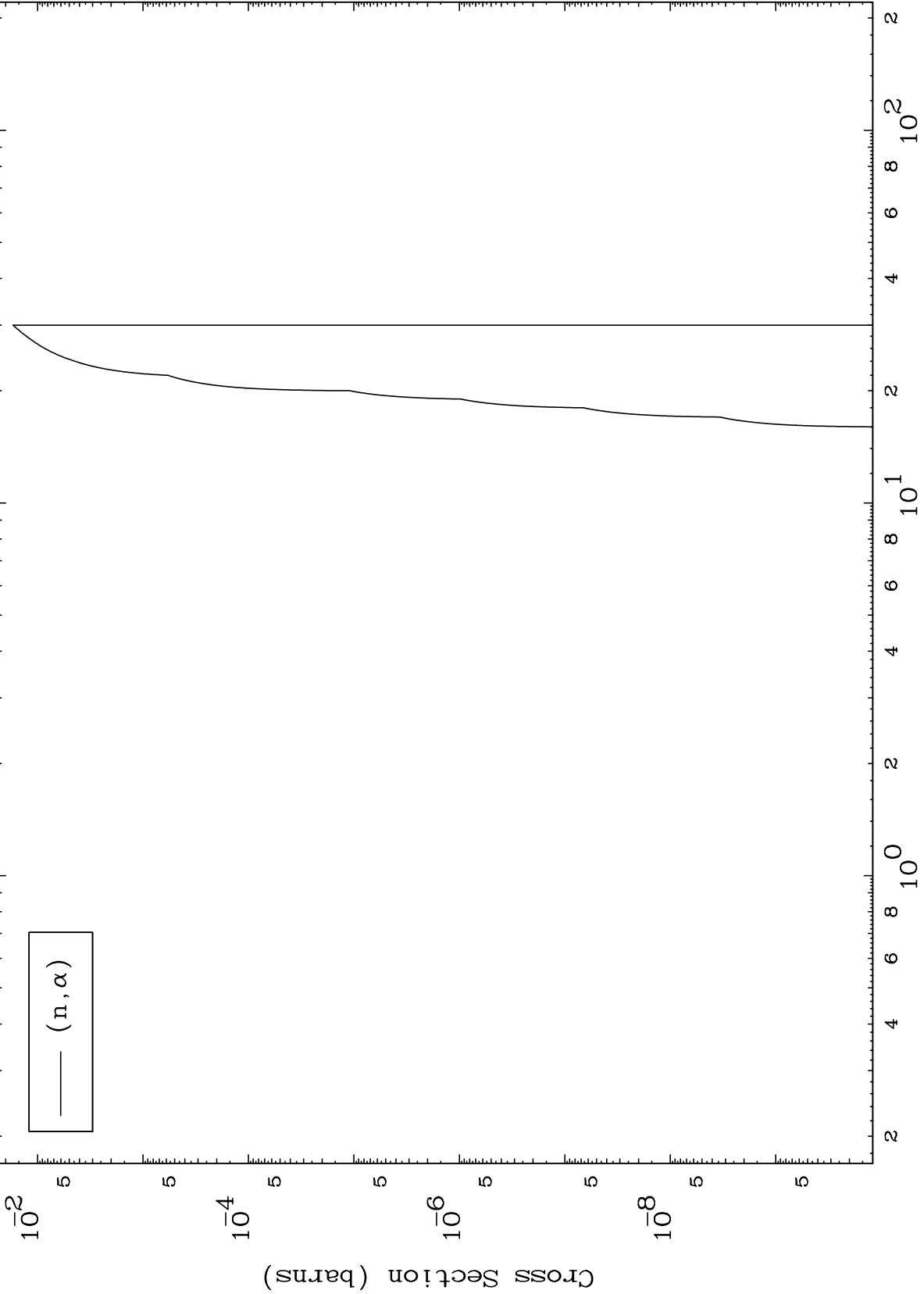
92-U -227



MAT 9204

( $\alpha, \alpha$ ) Levels  
0 Kelvin Cross Sections

92-U -227



MAT 9204

Fission

92-U -227

Radionuclide Production Cross Section

