

Program EVALPLOT
(Version 2021-1)

by

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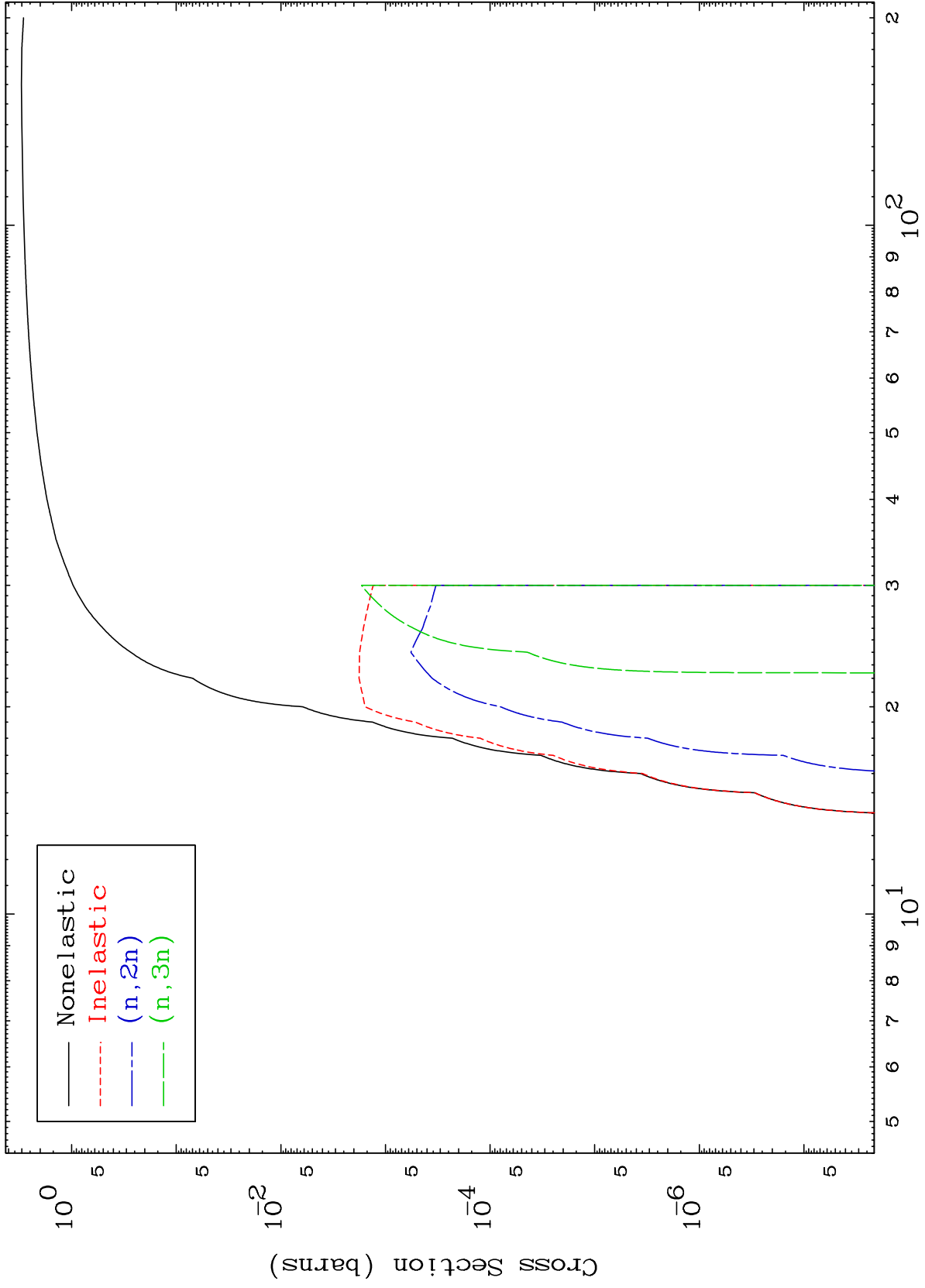
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9243

0 Kelvin Cross Sections

92-U -240



Incident Energy (MeV)

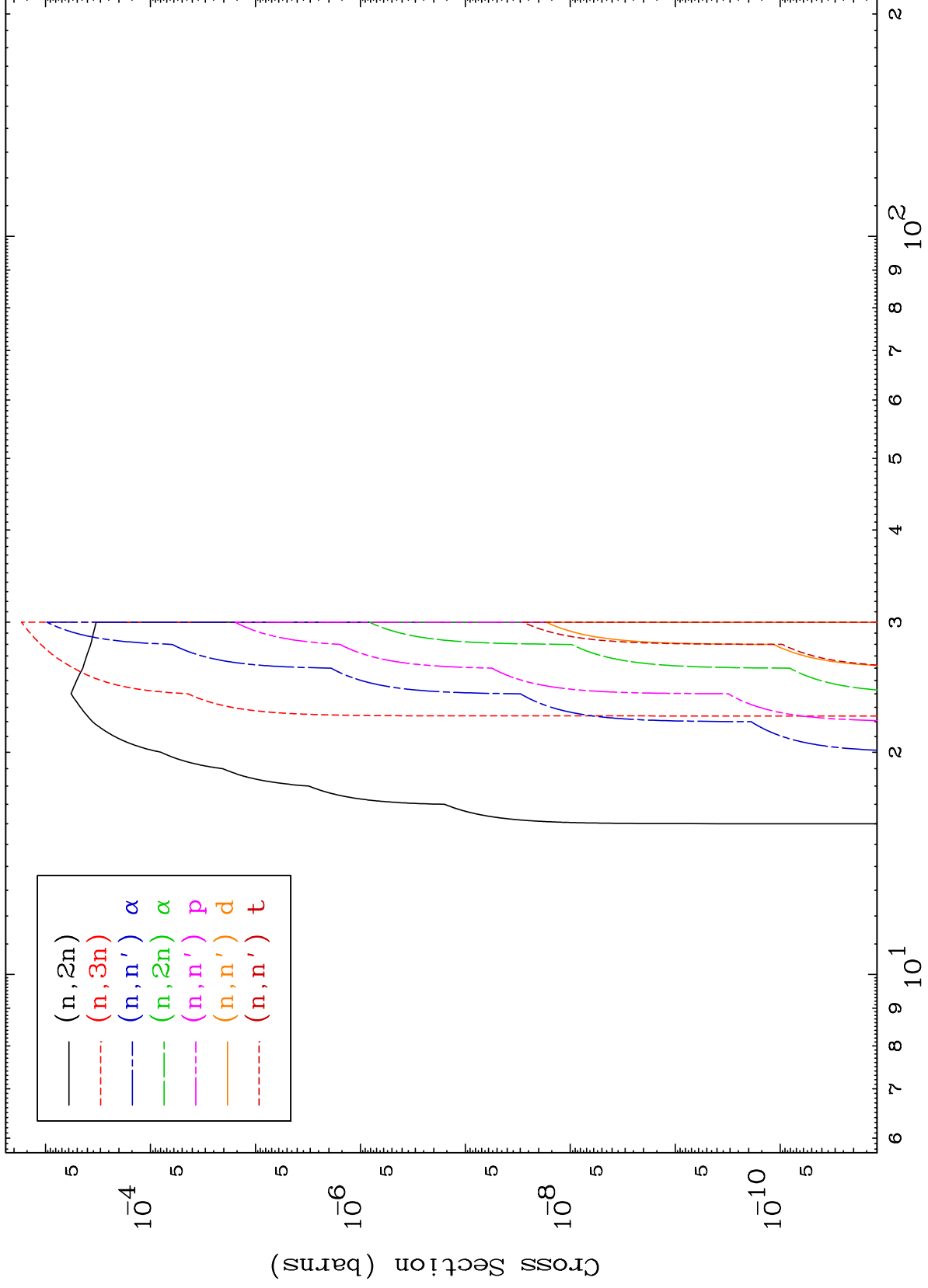
92-U -240

1

MAT 9243

α Neutron Absorption
0 Kelvin Cross Sections

92-U -240



2

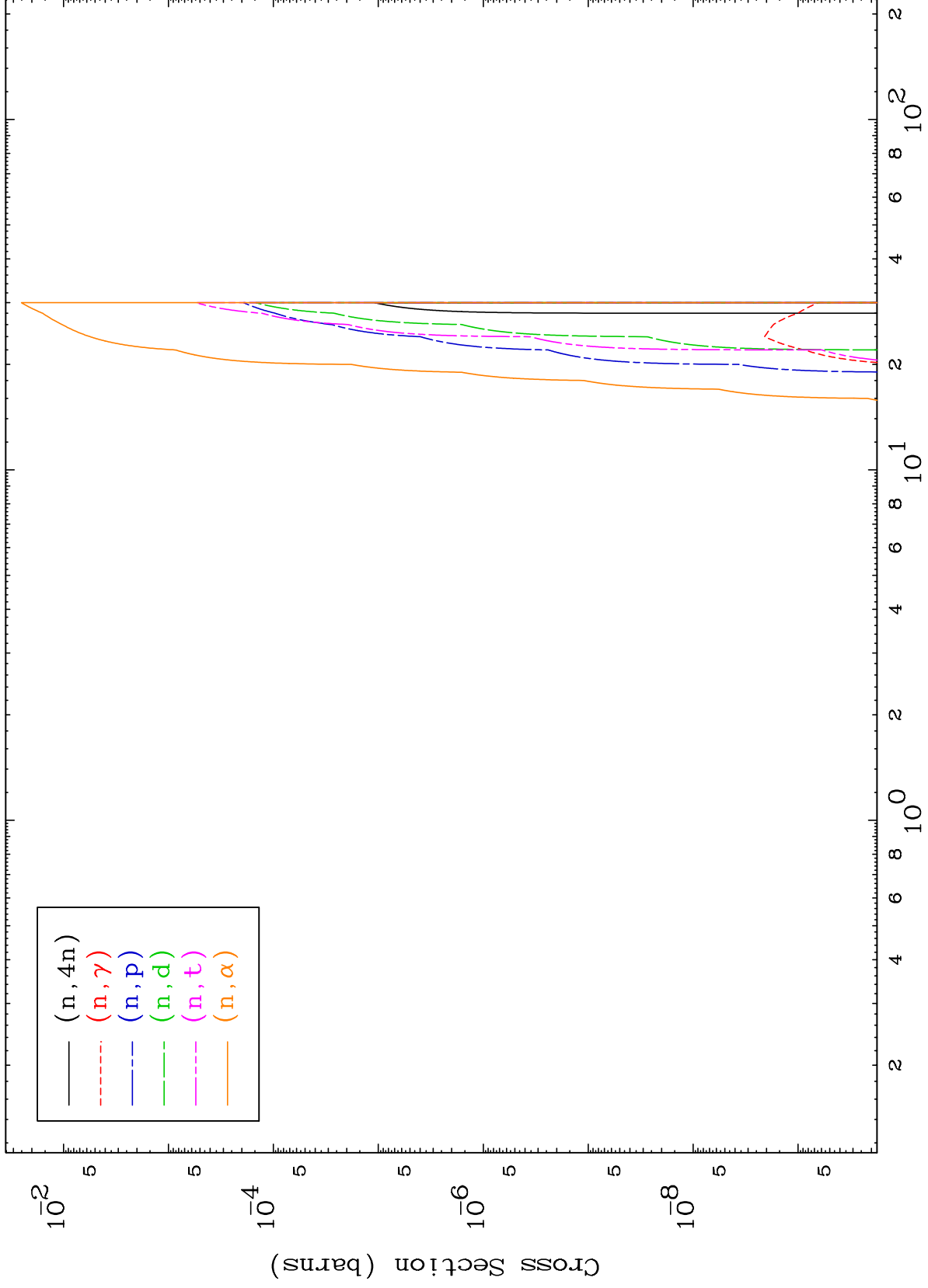
Incident Energy (MeV)

92-U -240

MAT 9243

α Neutron Absorption
0 Kelvin Cross Sections

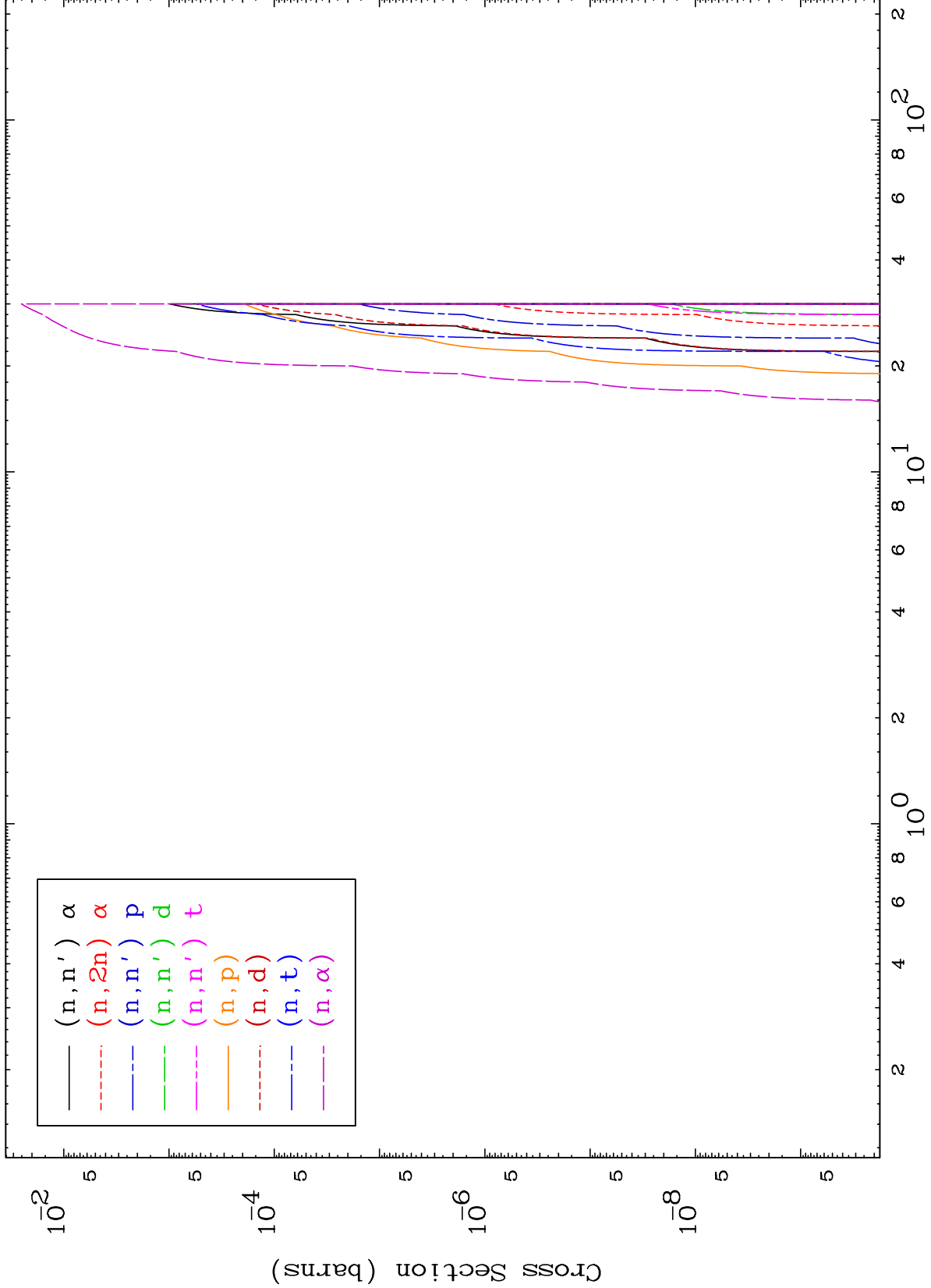
92-U -240



MAT 9243

α Charged Particle
0 Kelvin Cross Sections

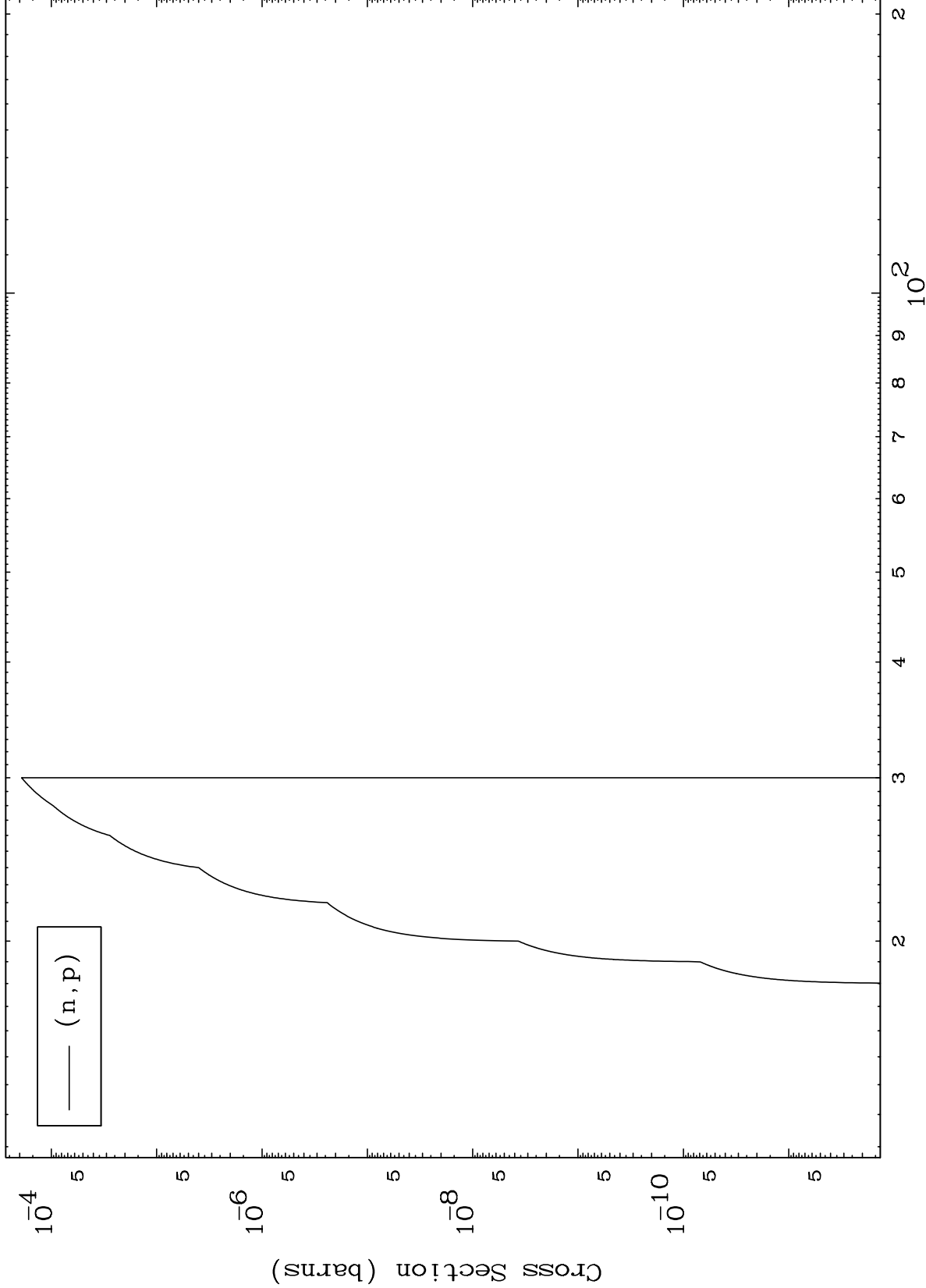
92-U -240



MAT 9243

(α, p) Levels
0 Kelvin Cross Sections

92-U -240



5

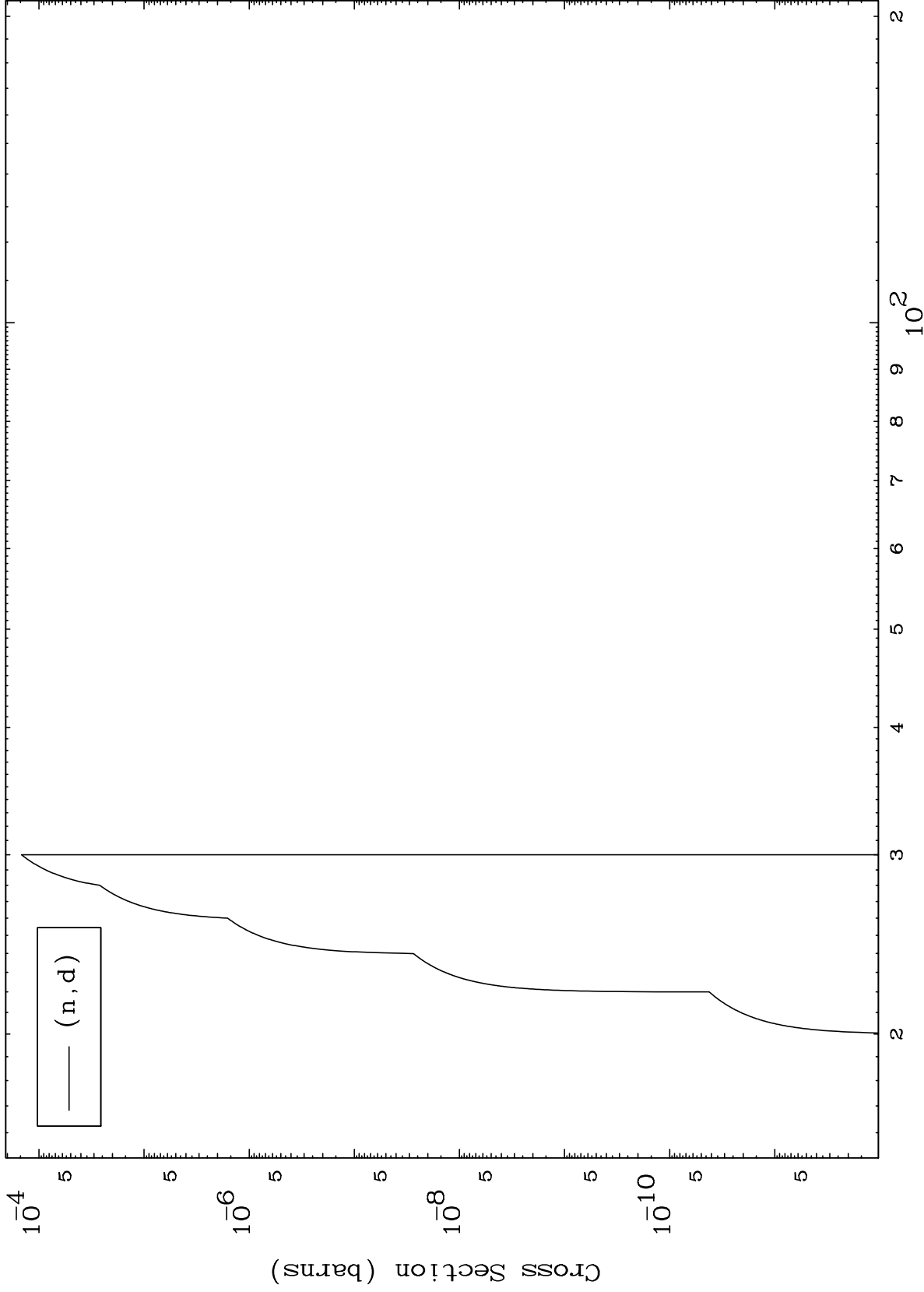
Incident Energy (MeV)

92-U -240

MAT 9243

(α, d) Levels
0 Kelvin Cross Sections

92-U -240



6

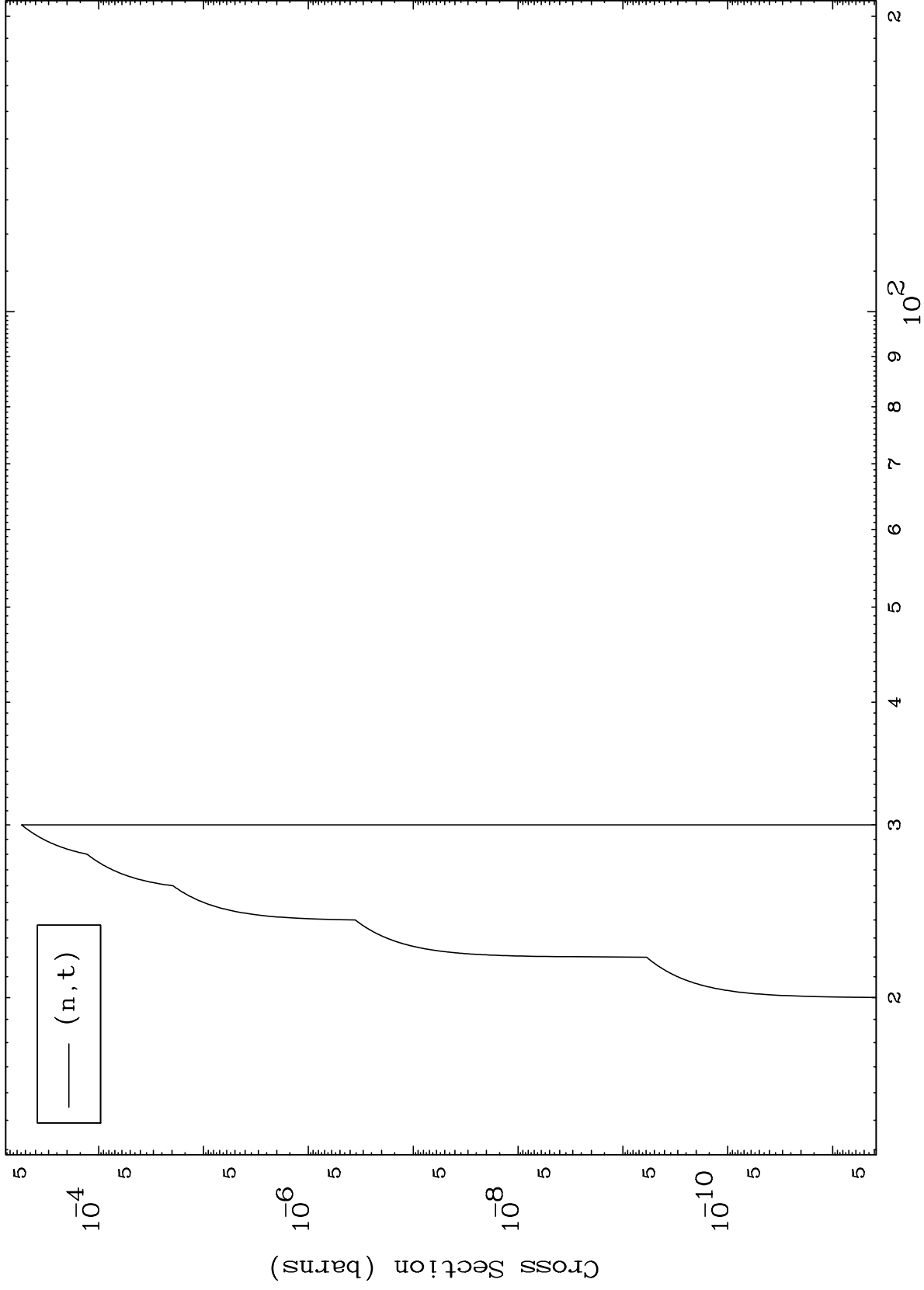
Incident Energy (MeV)

92-U -240

MAT 9243

(α, t) Levels
0 Kelvin Cross Sections

92-U -240



7

Incident Energy (MeV)

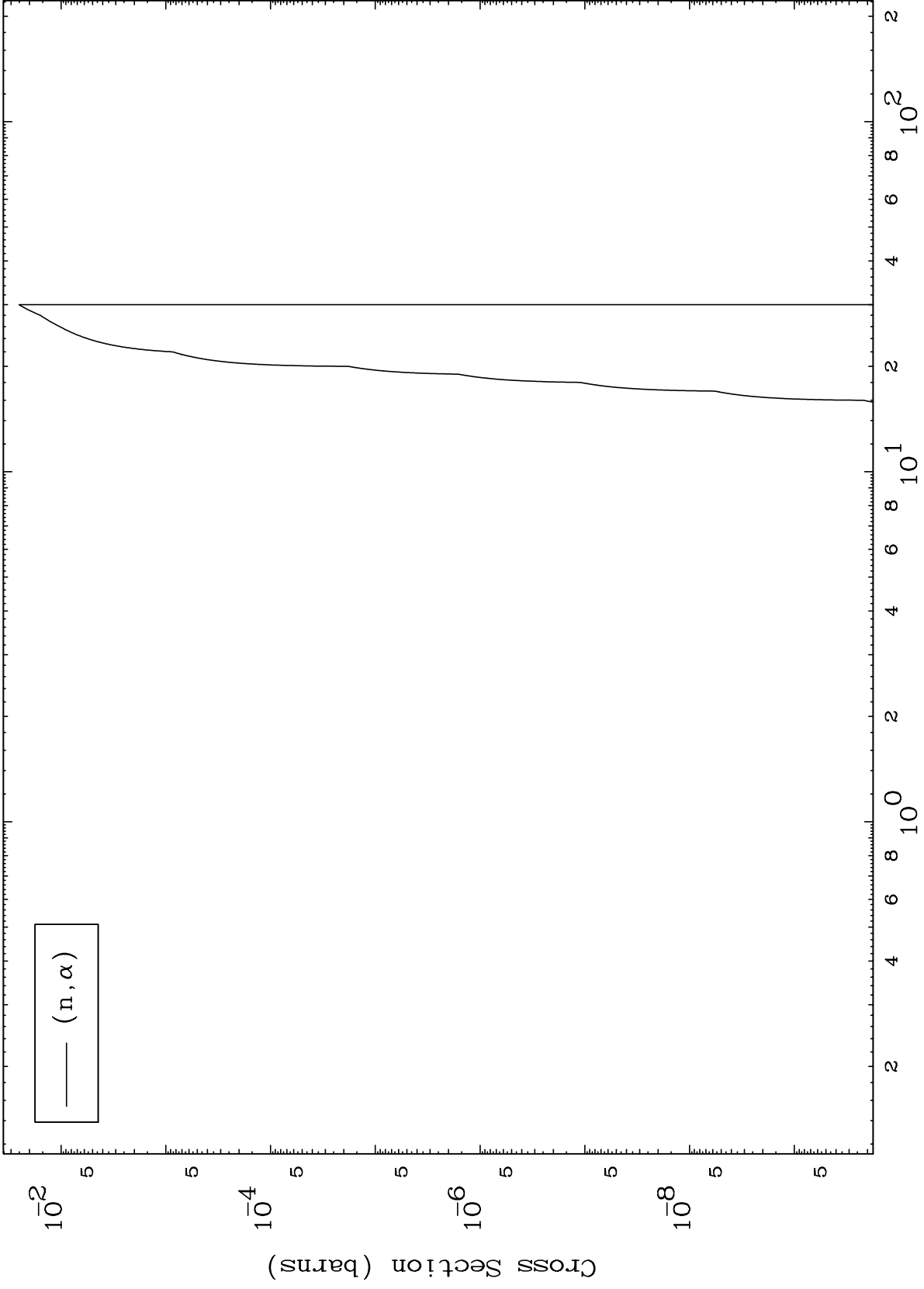
92-U -240

MAT 9243

(α, α) Levels

92-U -240

0 Kelvin Cross Sections

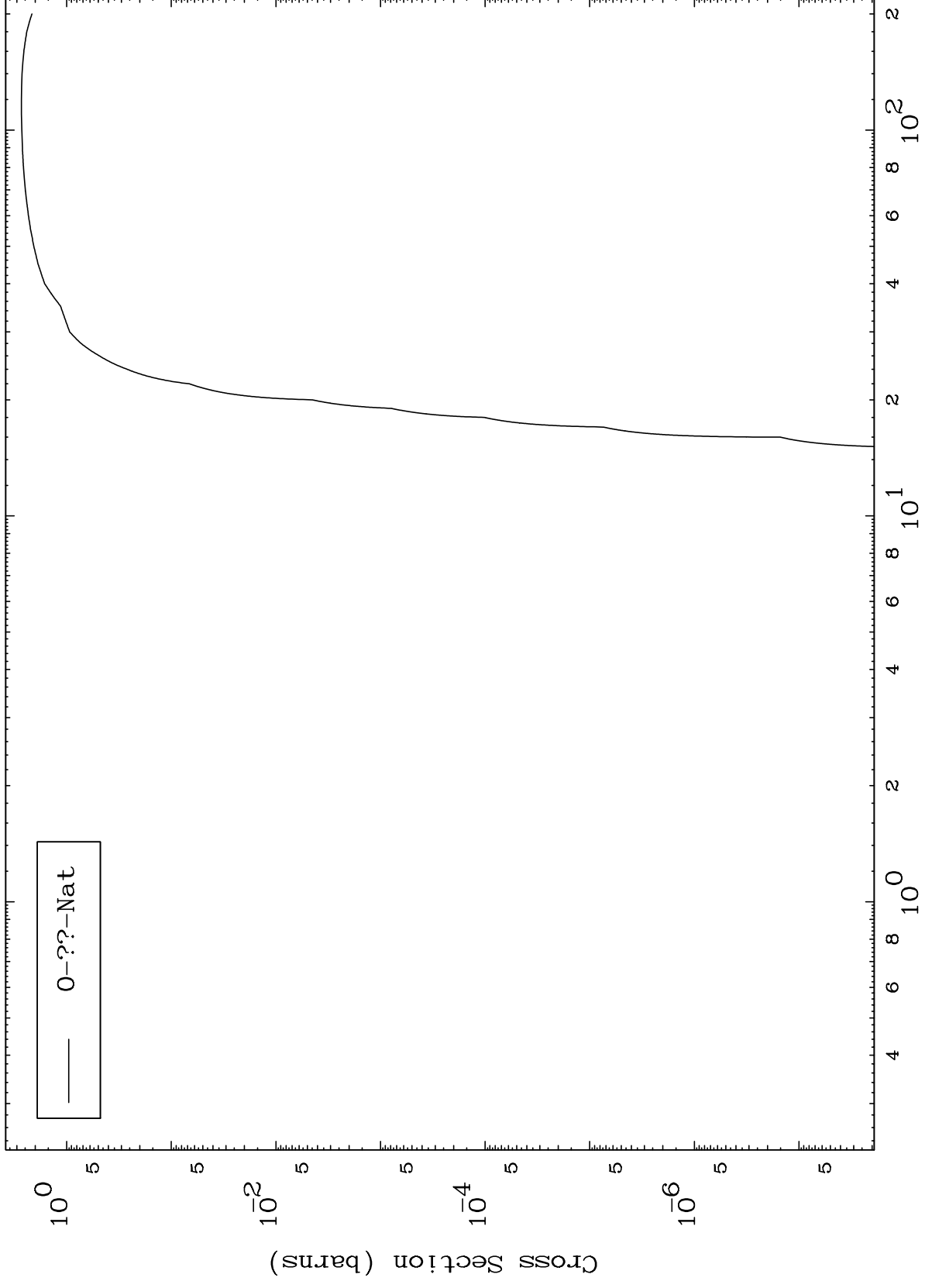


MAT 9243

Fission

92-U -240

Radionuclide Production Cross Section

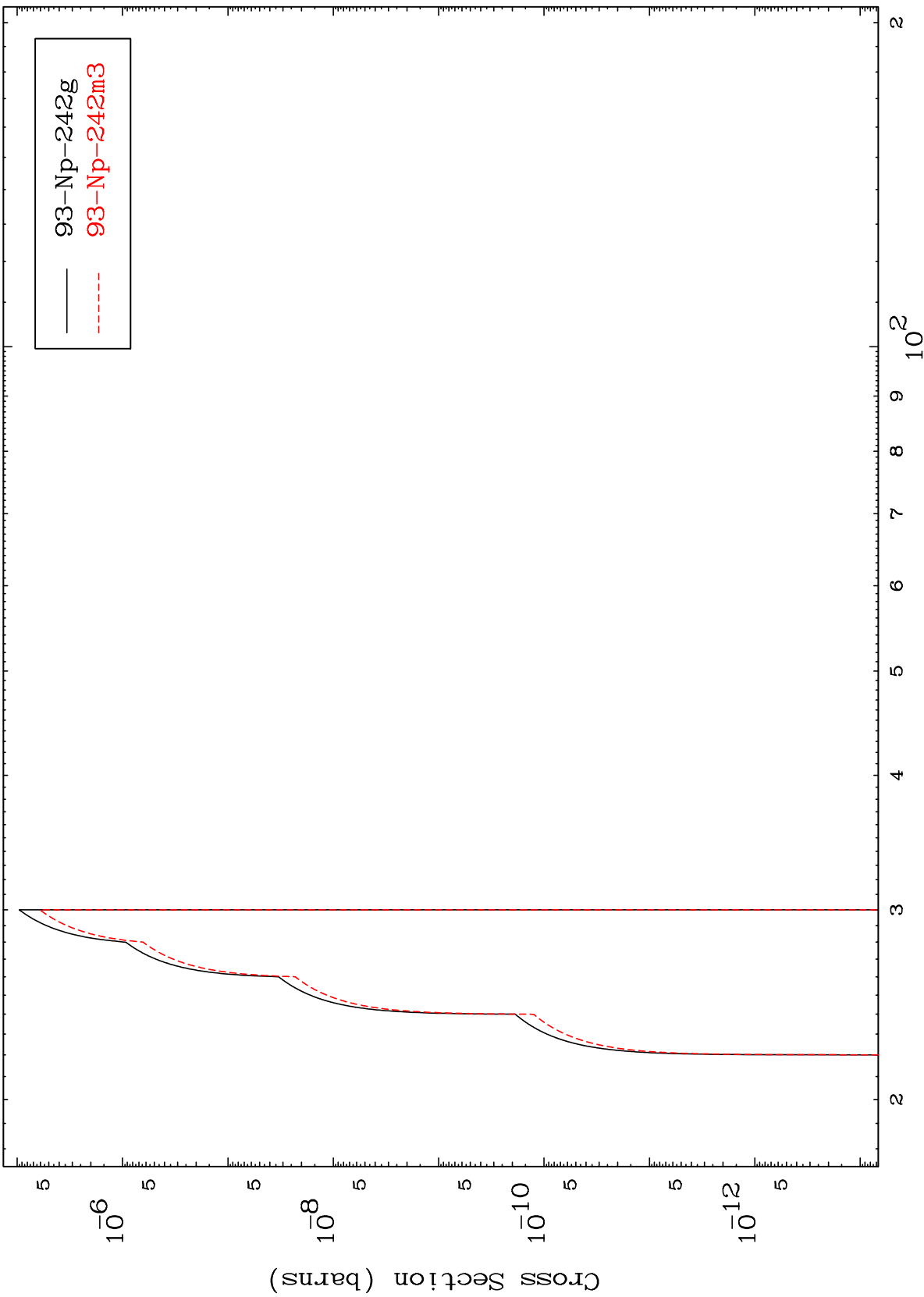


MAT 9243

(n,n') p

92-U -240

Radionuclide Production Cross Section



10

Incident Energy (MeV)

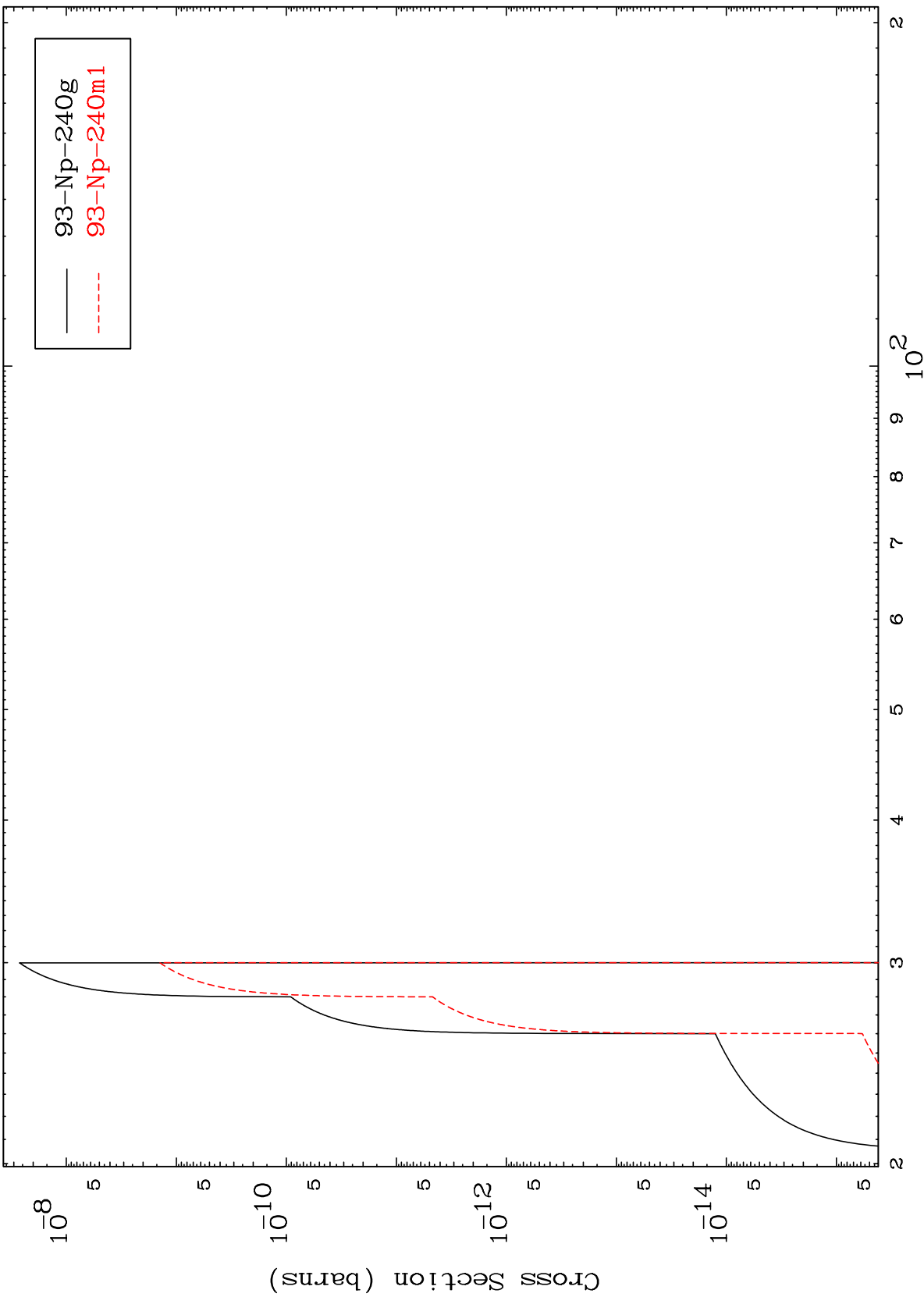
92-U -240

MAT 9243

(n,n') t

92-U -240

Radionuclide Production Cross Section



11

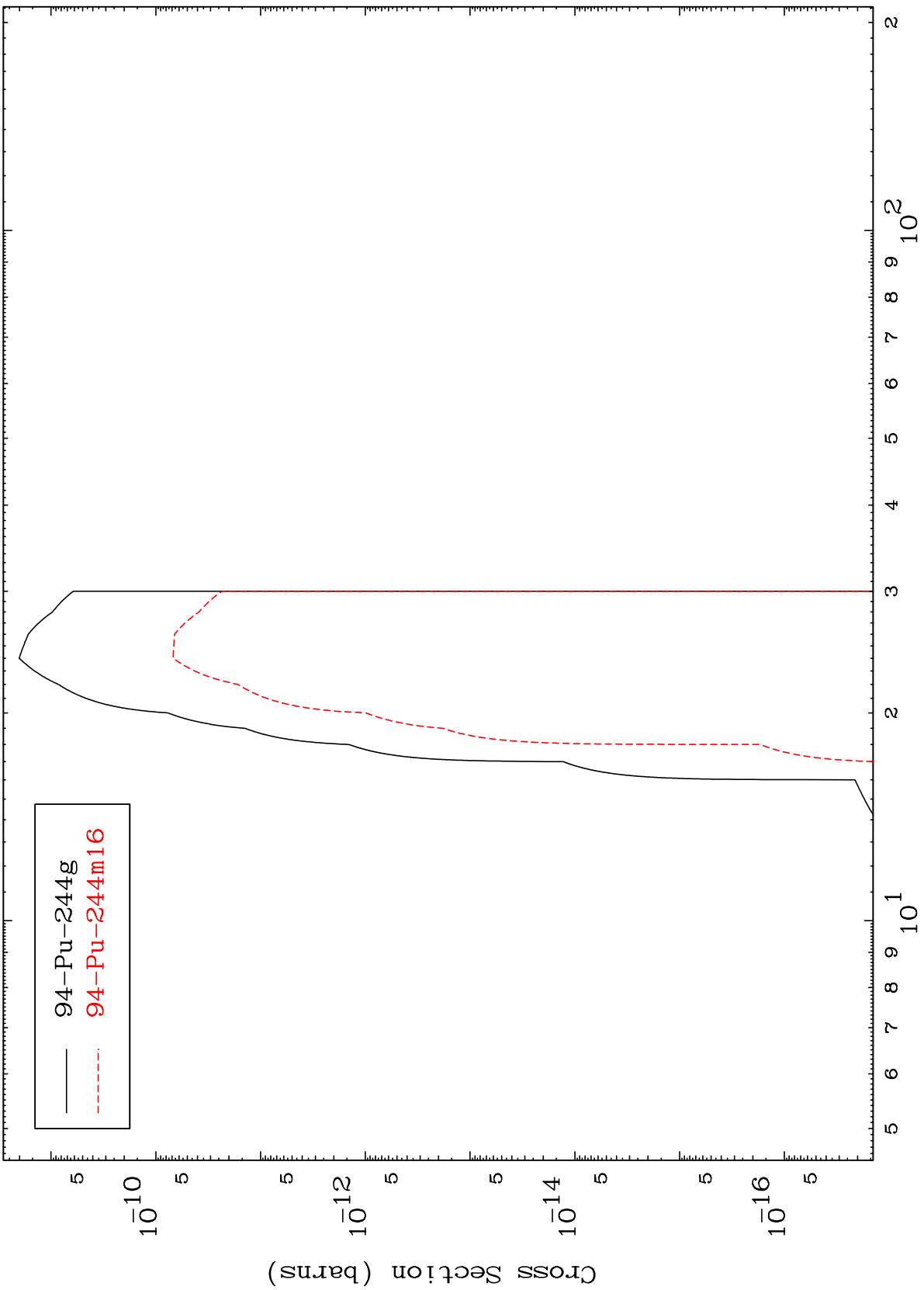
Incident Energy (MeV)

92-U -240

MAT 9243

92-U -240

Radionuclide Production Cross Section (n,γ)



12

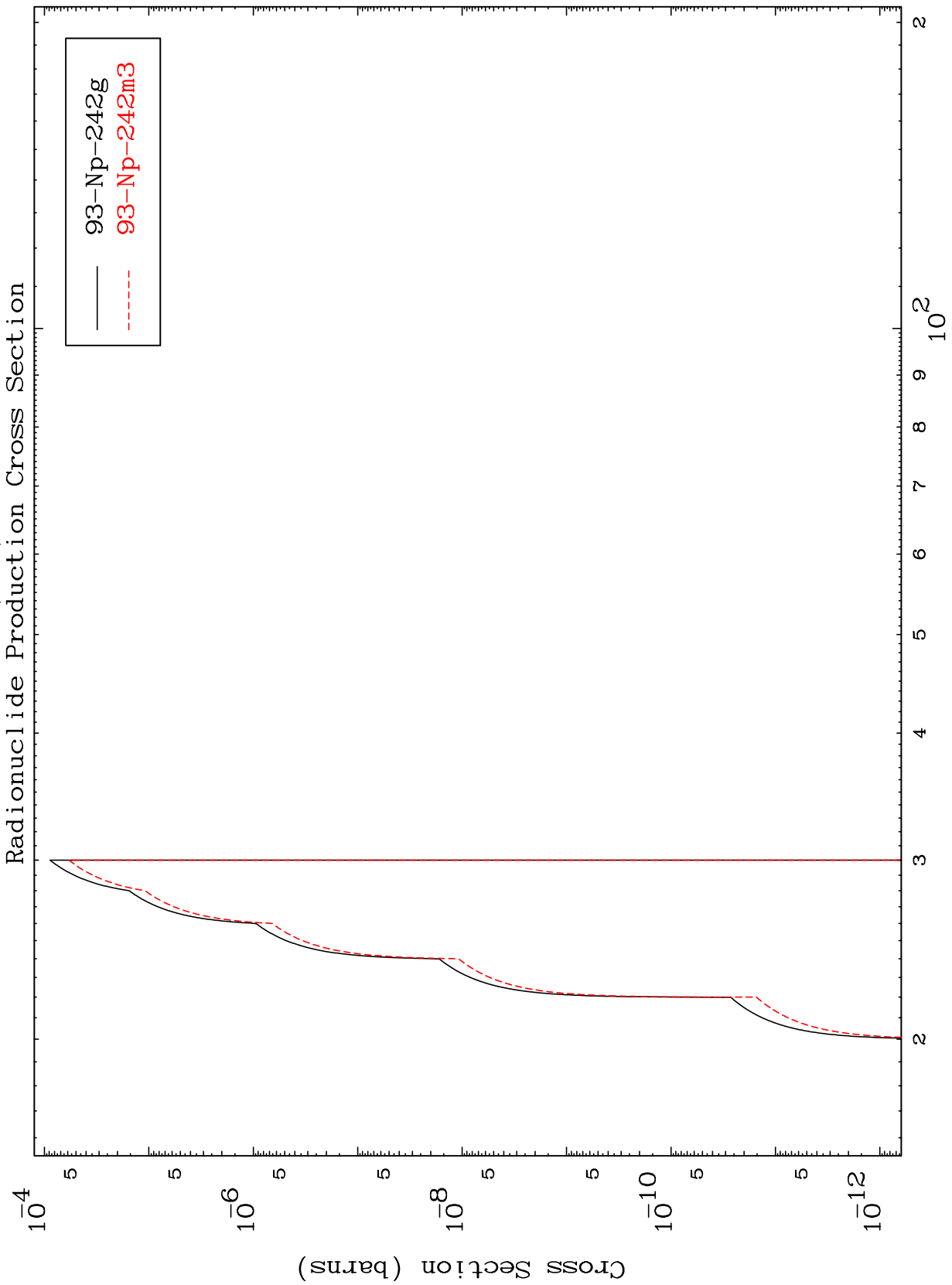
Incident Energy (MeV)

92-U -240

MAT 9243

92-U -240

(n,d)
Radionuclide Production Cross Section



13

92-U -240

Incident Energy (MeV)