

Program EVALPLOT  
(Version 2021-1)

by

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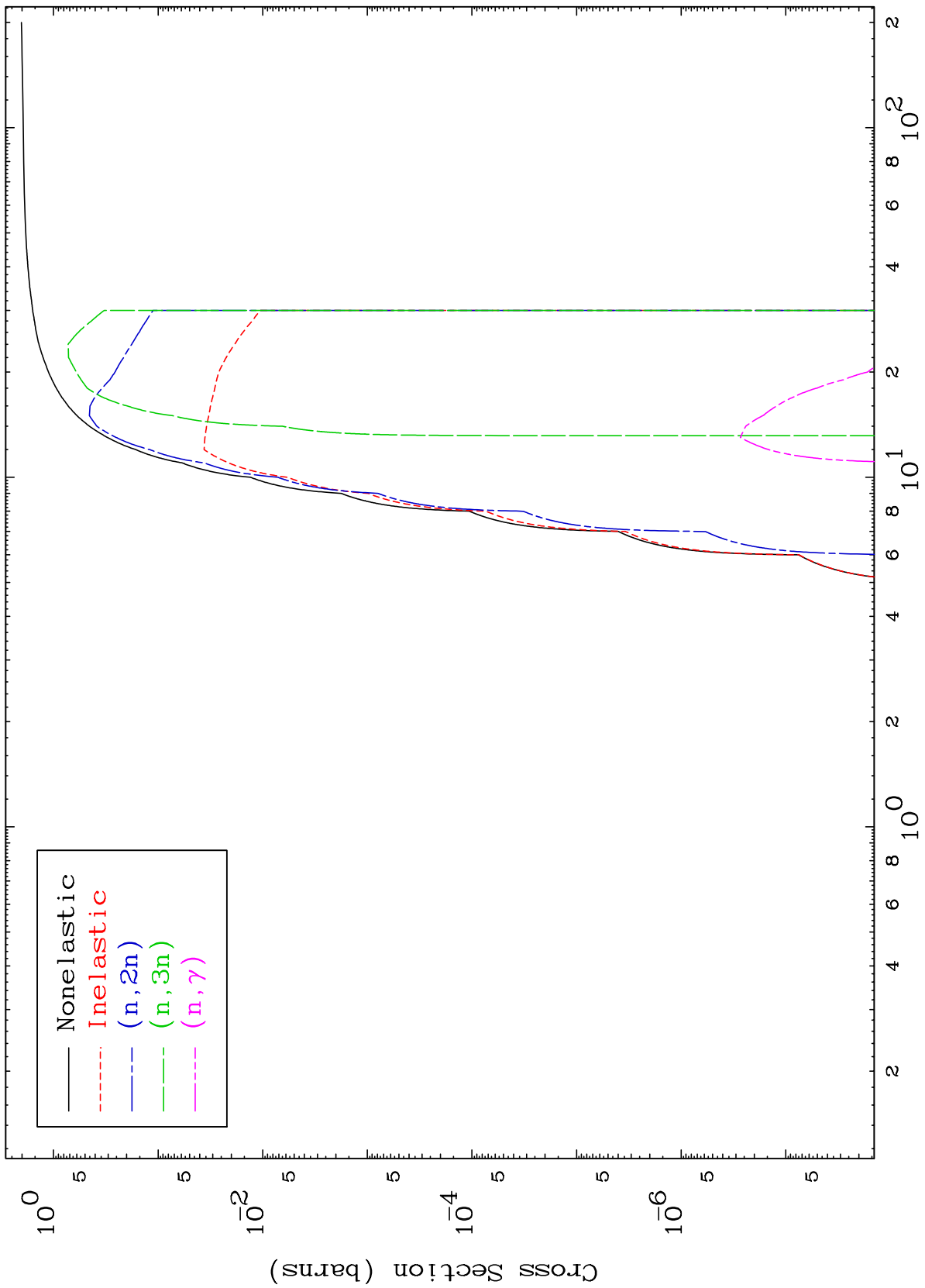
Press Mouse Button to Start

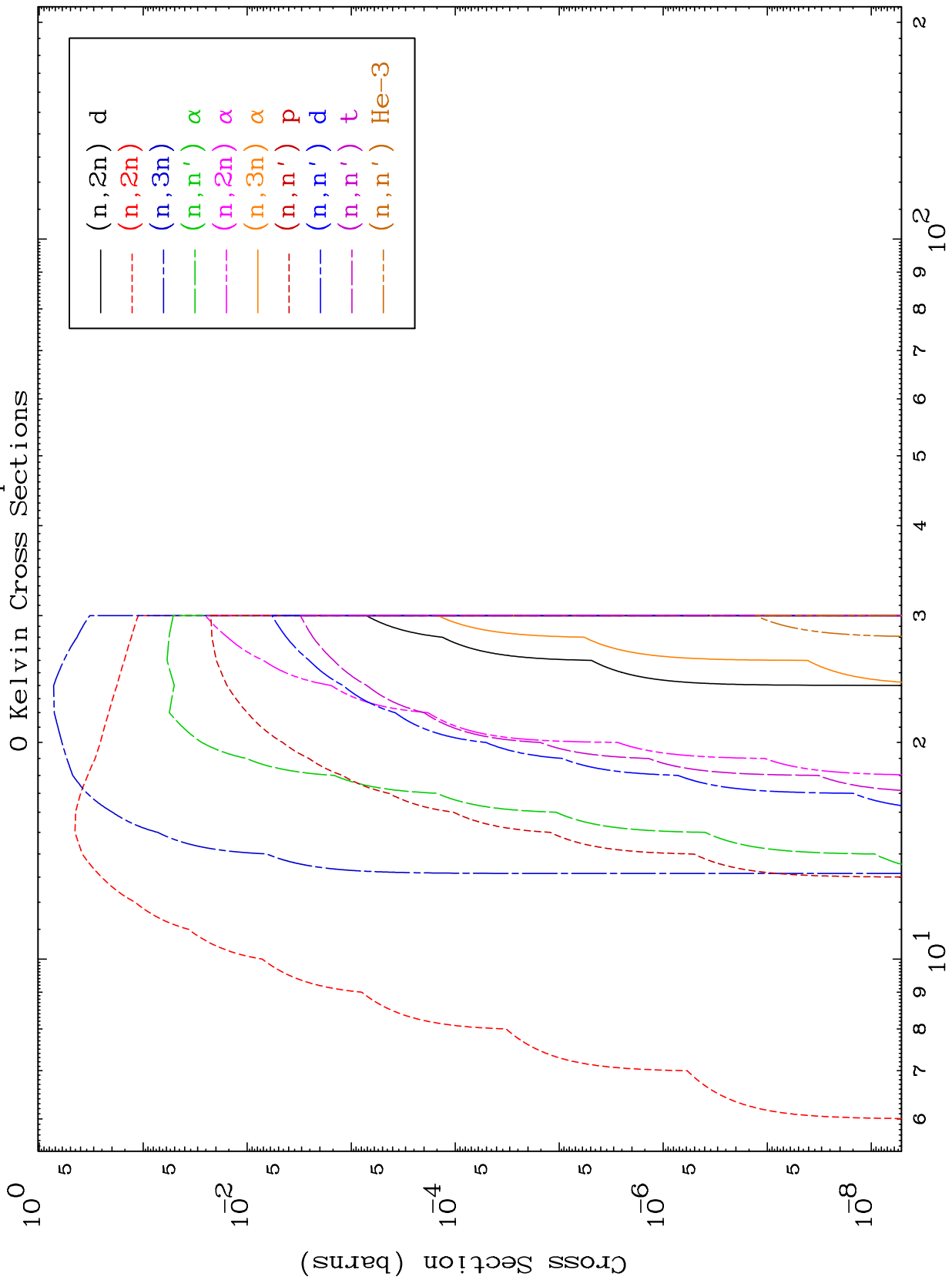
MAT 3955

$\alpha$  Major

0 Kelvin Cross Sections

39-Y -99

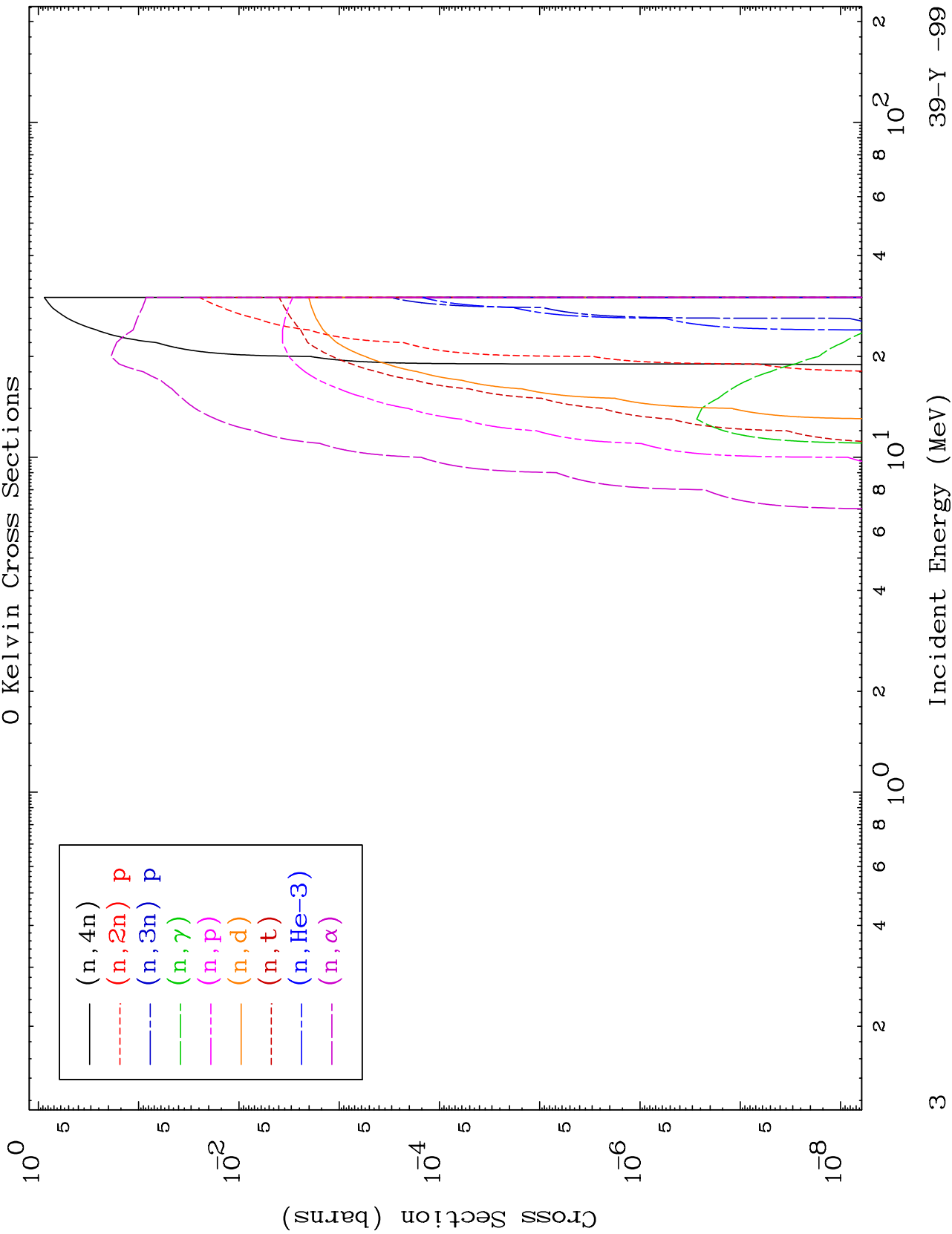


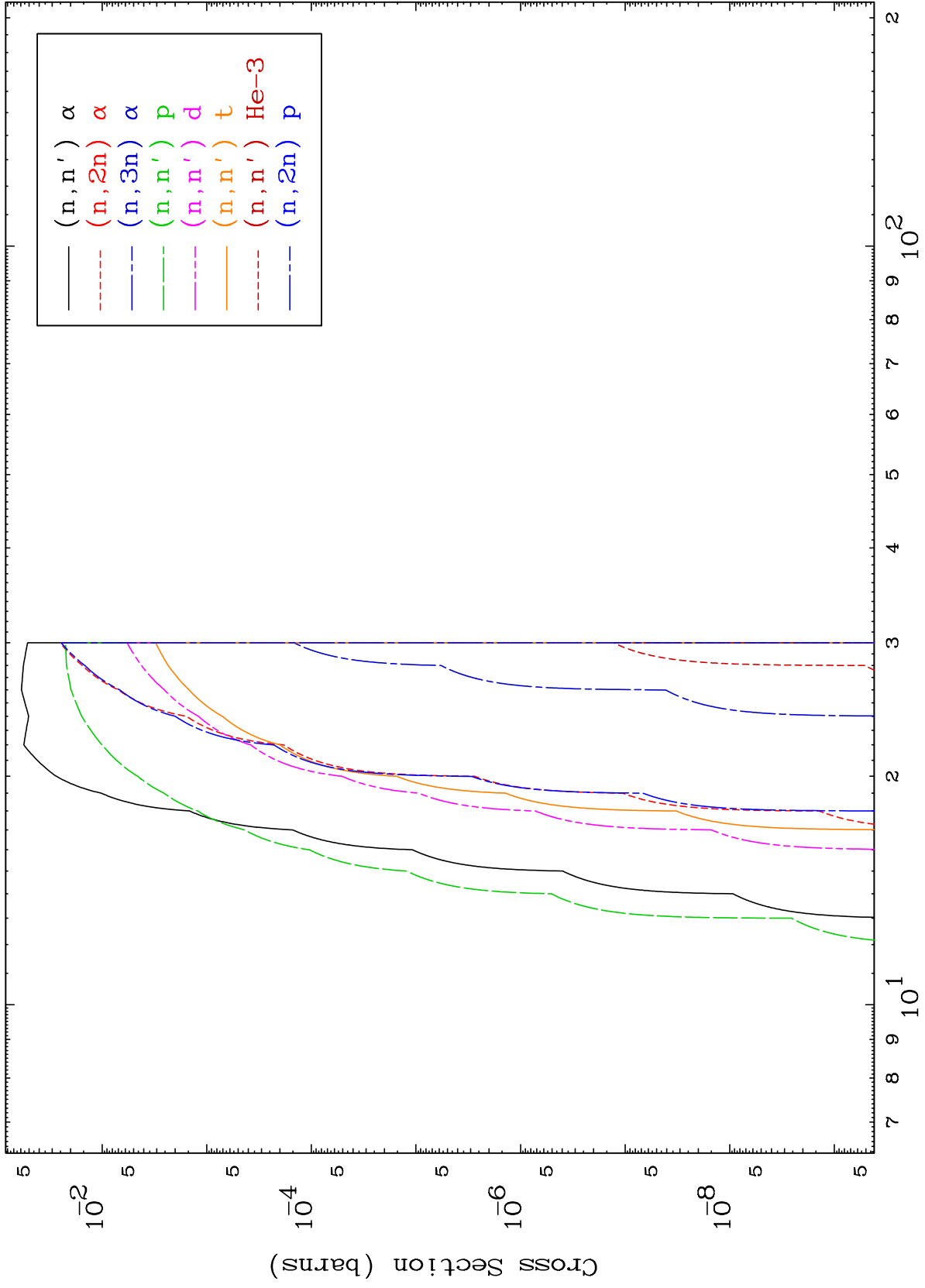


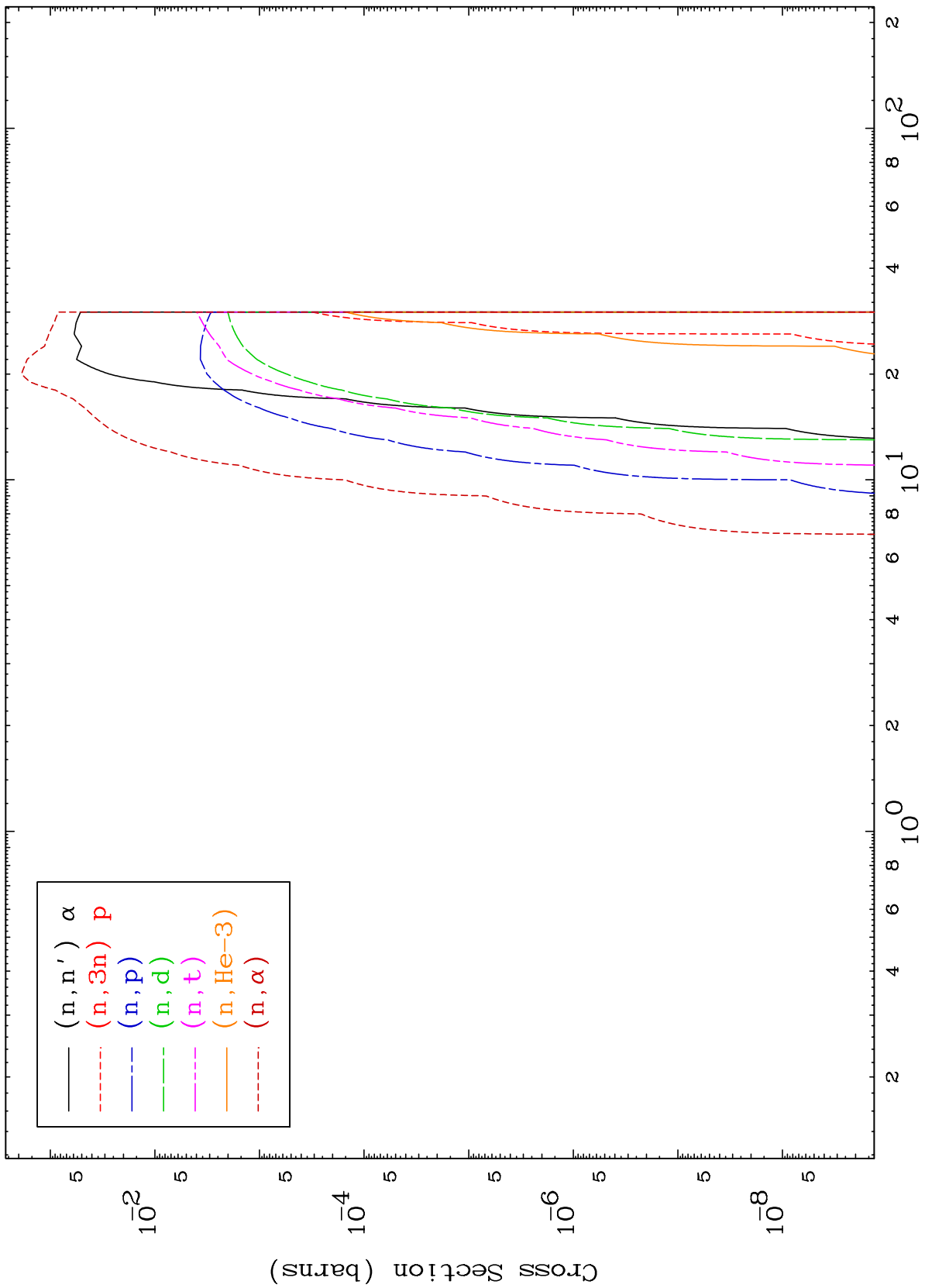
MAT 3955

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

39-Y -99





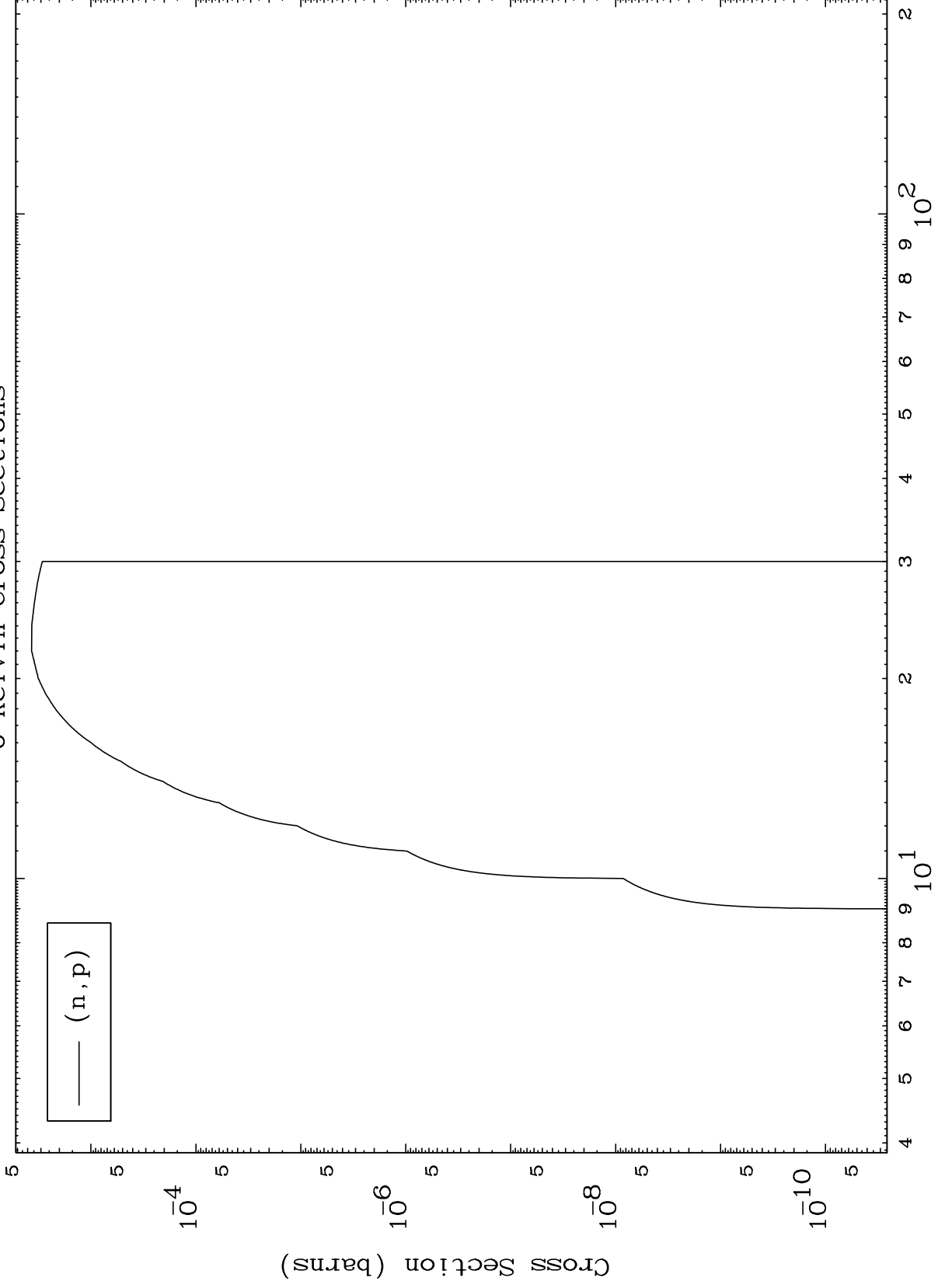


MAT 3955

( $\alpha, p$ ) Levels

39-Y -99

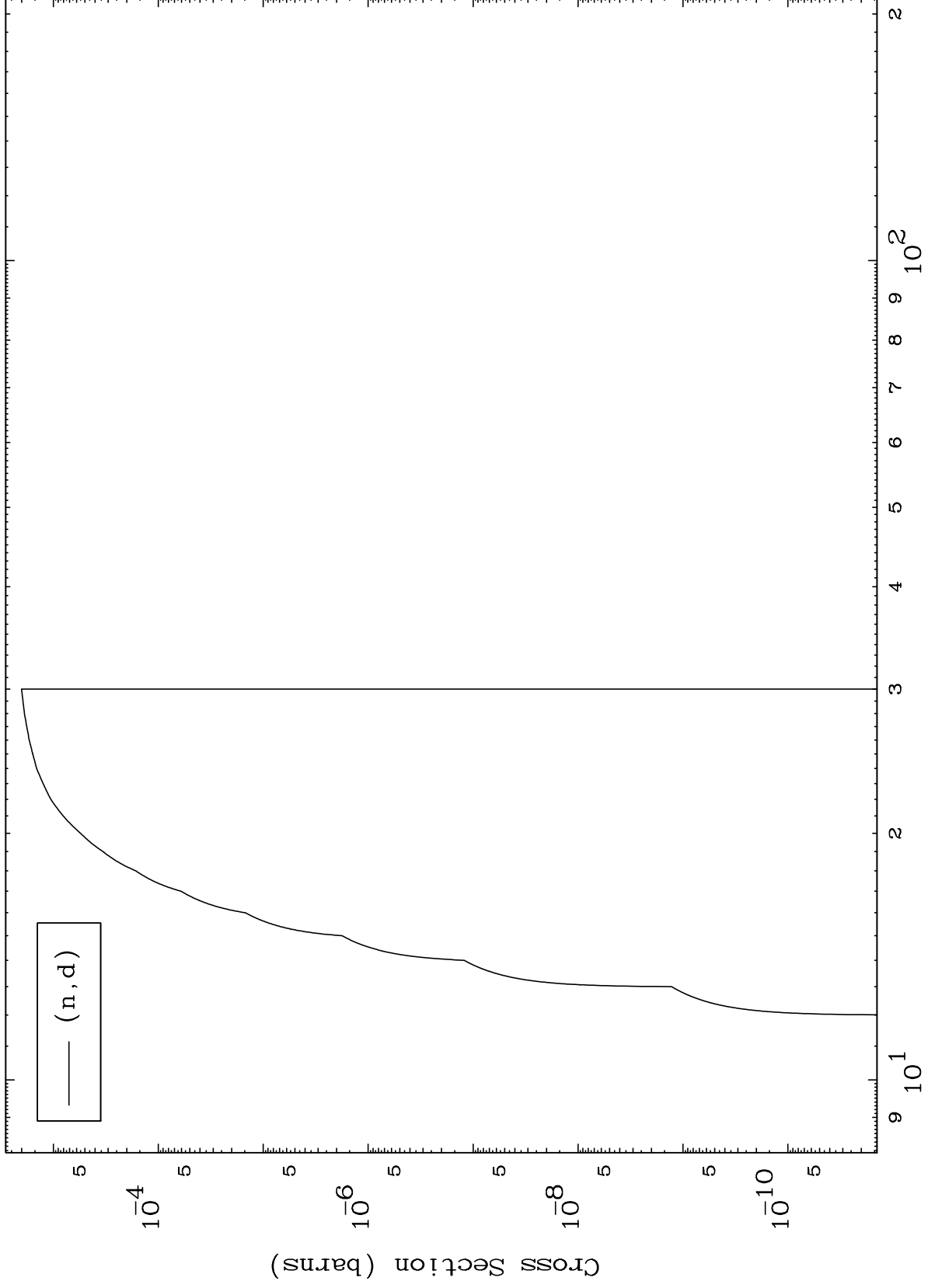
0 Kelvin Cross Sections

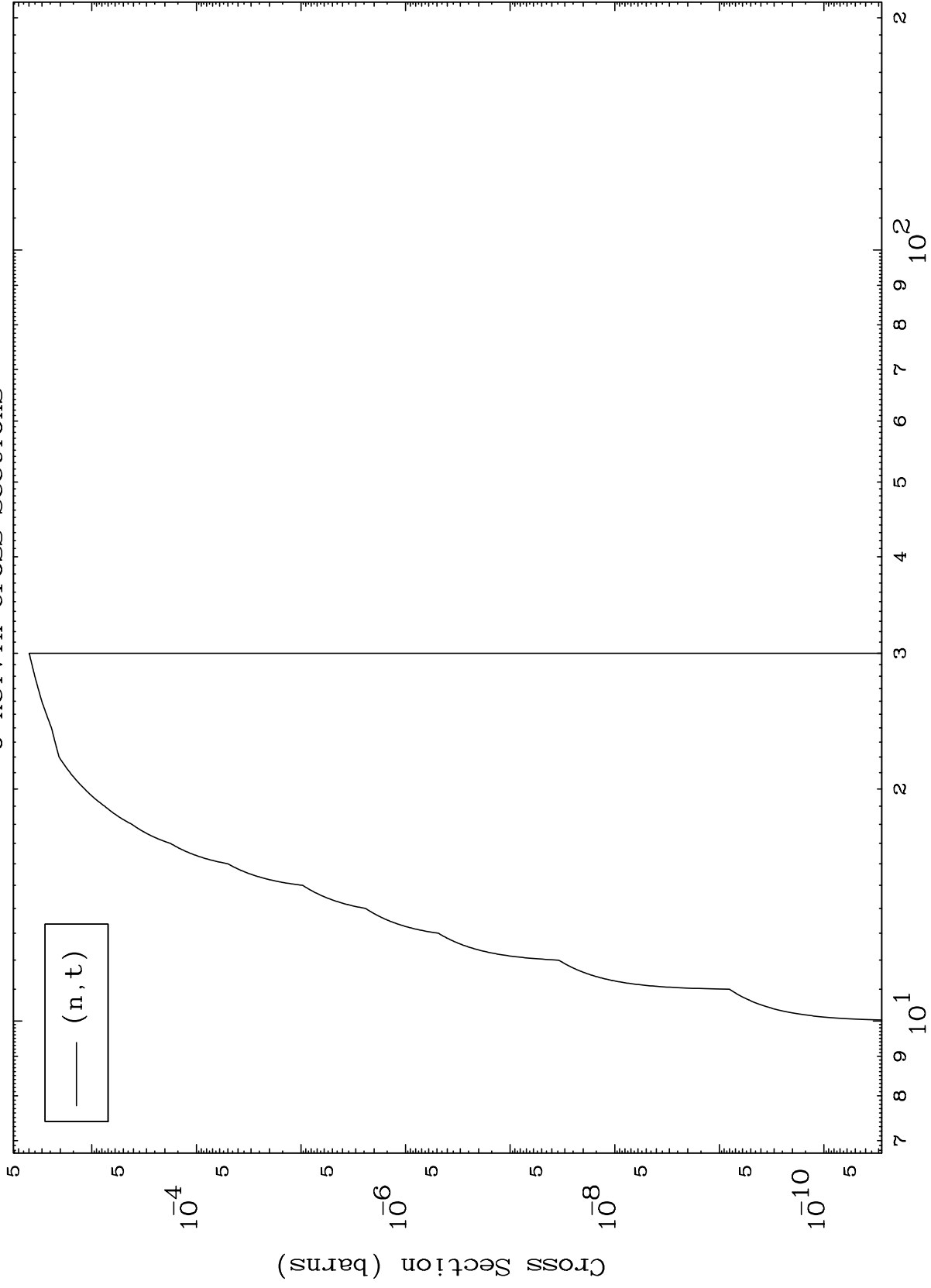


6

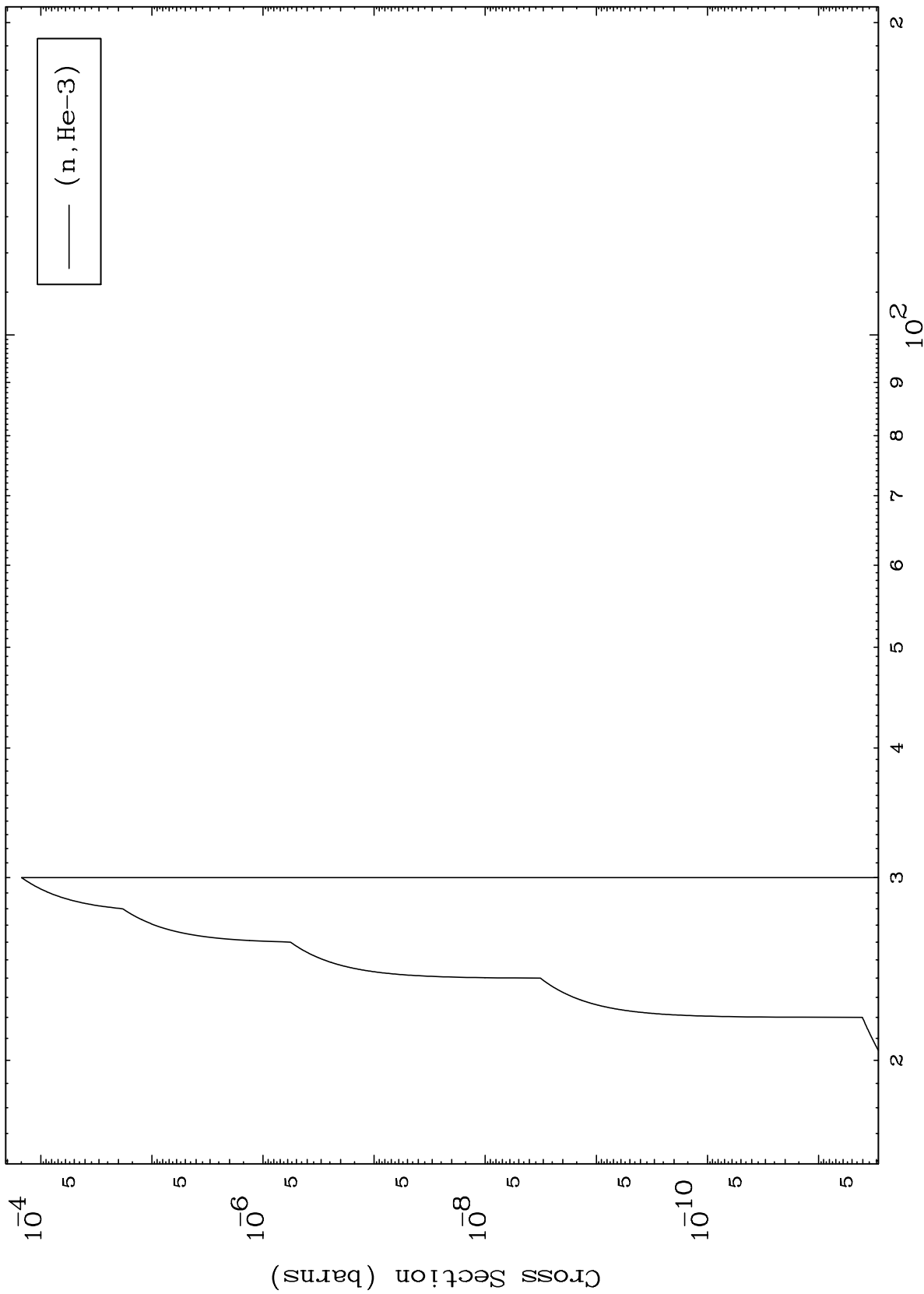
Incident Energy (MeV)

39-Y -99





( $\alpha, \text{He3}$ ) Levels  
0 Kelvin Cross Sections

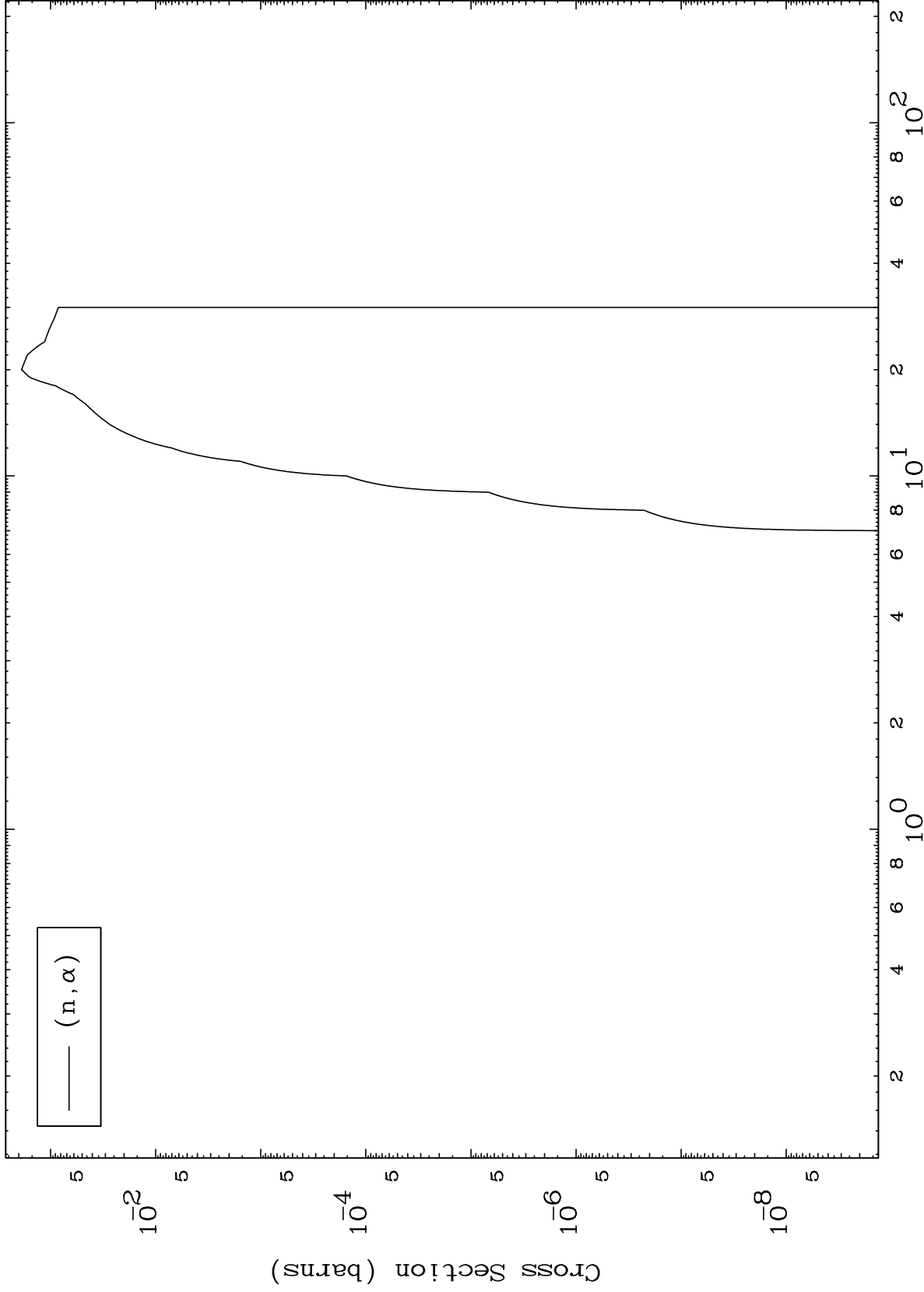


MAT 3955

( $\alpha, \alpha$ ) Levels

39-Y -99

0 Kelvin Cross Sections



10

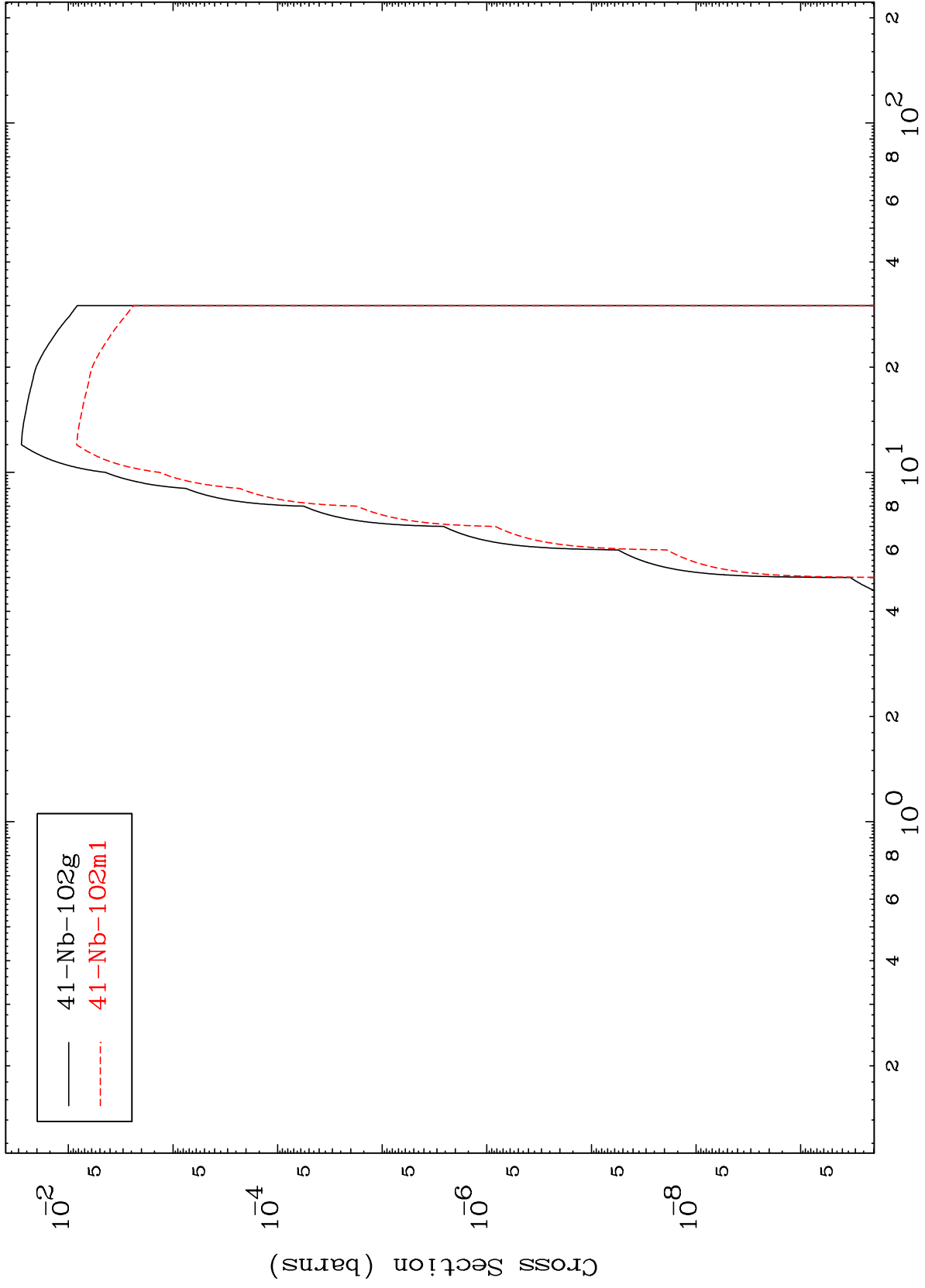
Incident Energy (MeV)

39-Y -99

MAT 3955

39-Y -99

Inelastic  
Radionuclide Production Cross Section



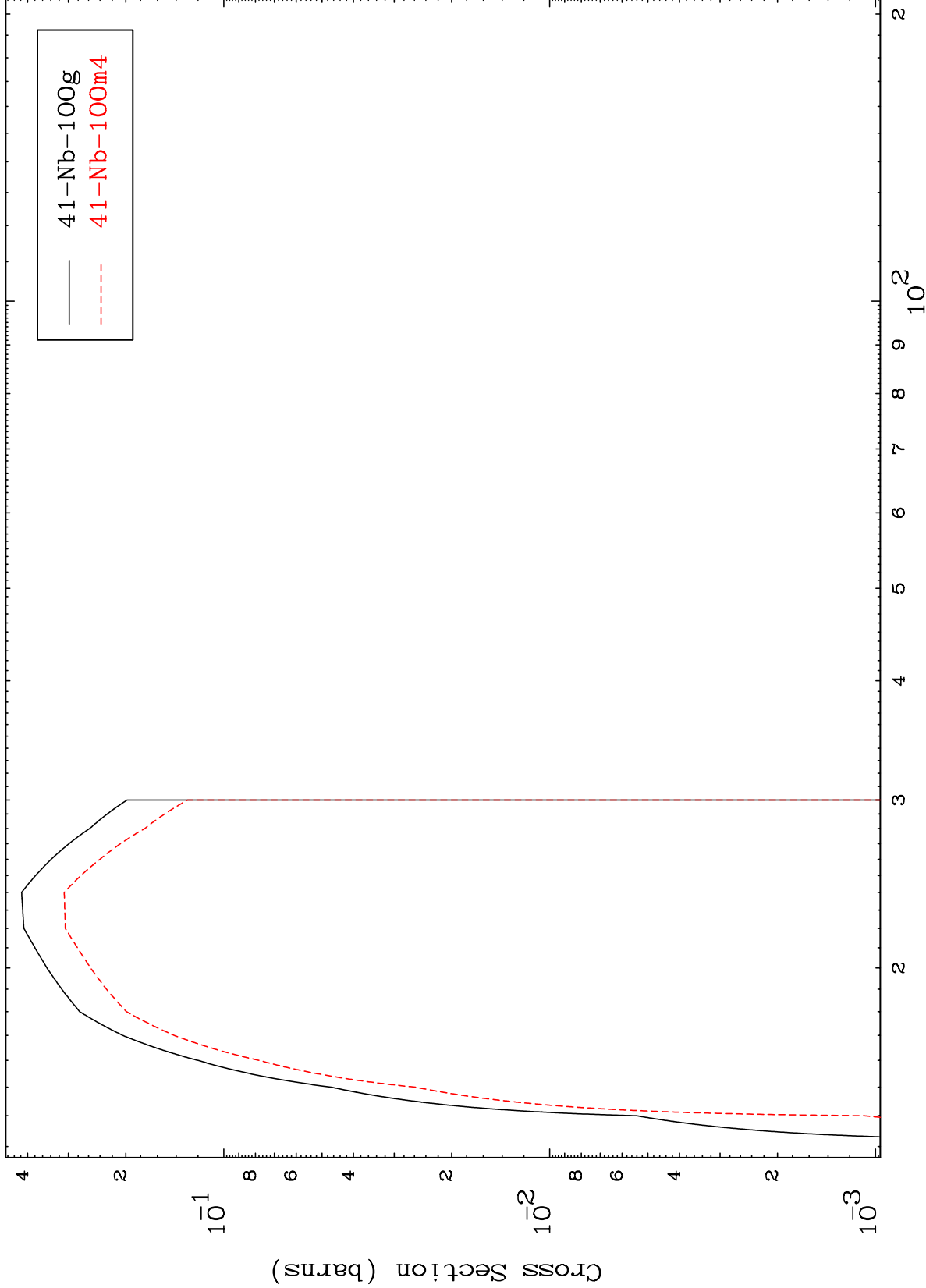
— 41-Nb-102g  
- - - 41-Nb-102m1

MAT 3955

(n,3n)

39-Y -99

Radionuclide Production Cross Section



12

Incident Energy (MeV)

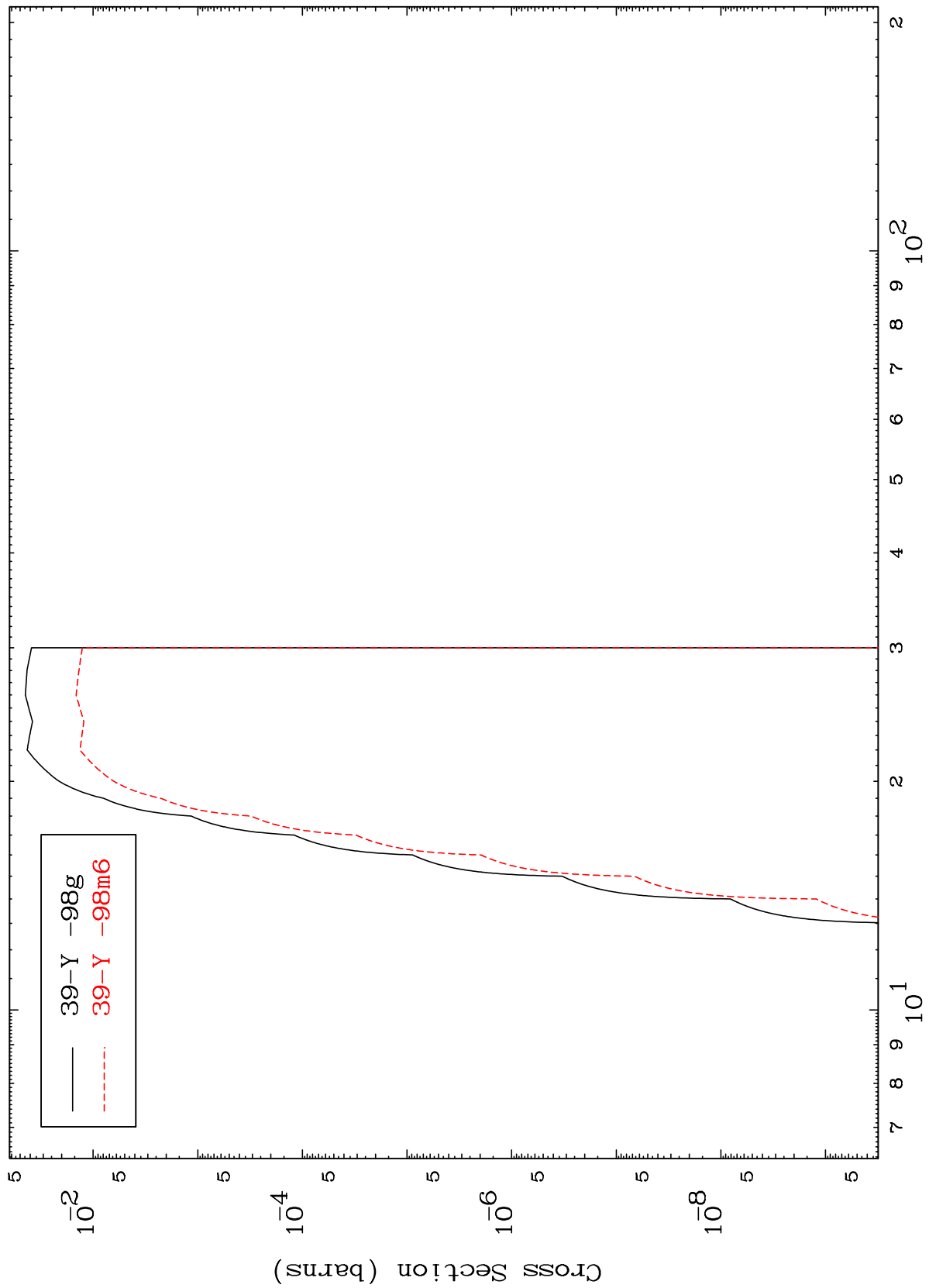
39-Y -99

MAT 3955

$(n, n') \alpha$

39-Y -99

Radionuclide Production Cross Section



13

Incident Energy (MeV)

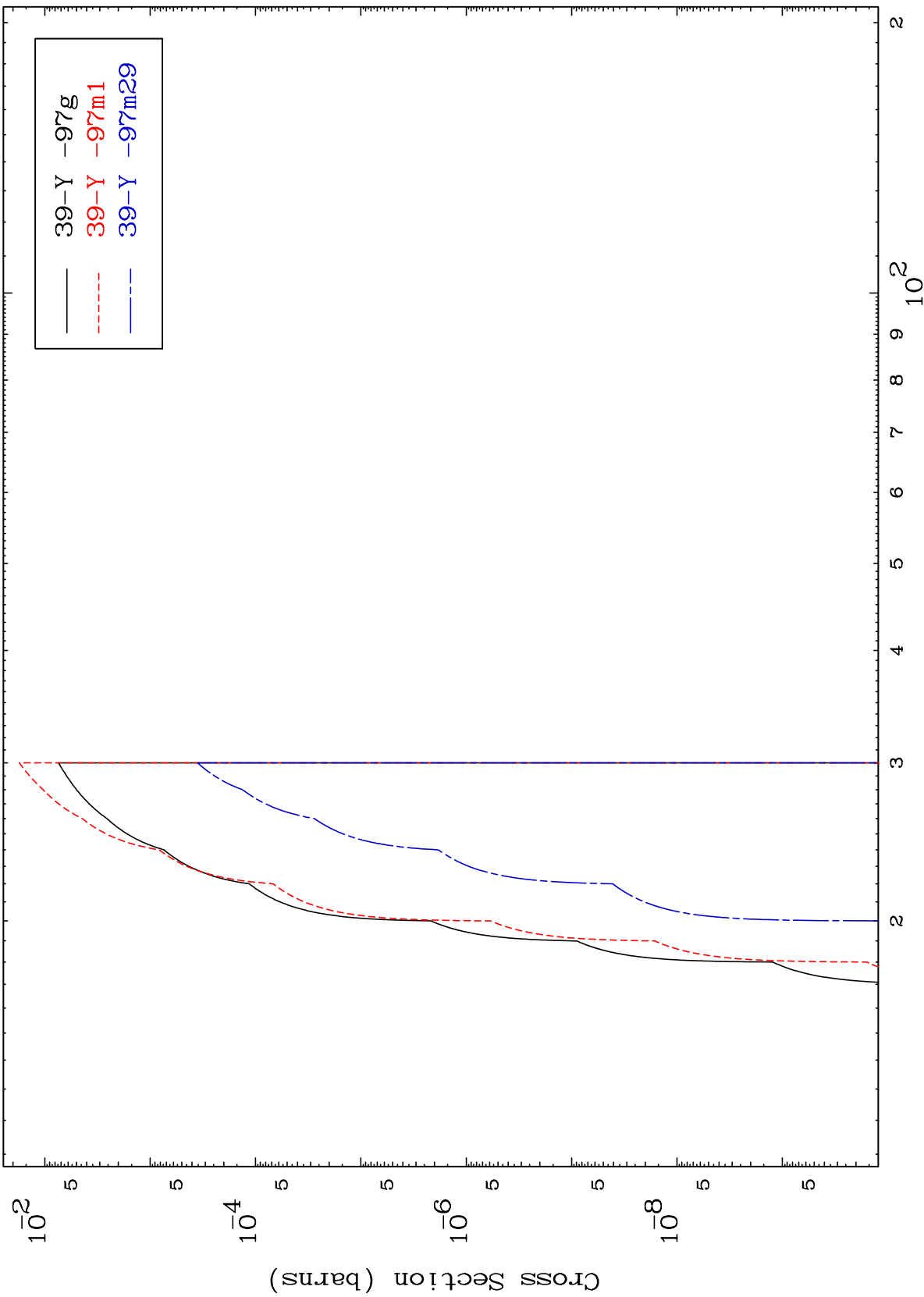
39-Y -99

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$(n,2n) \alpha$

39-Y -99

Radionuclide Production Cross Section



14

Incident Energy (MeV)

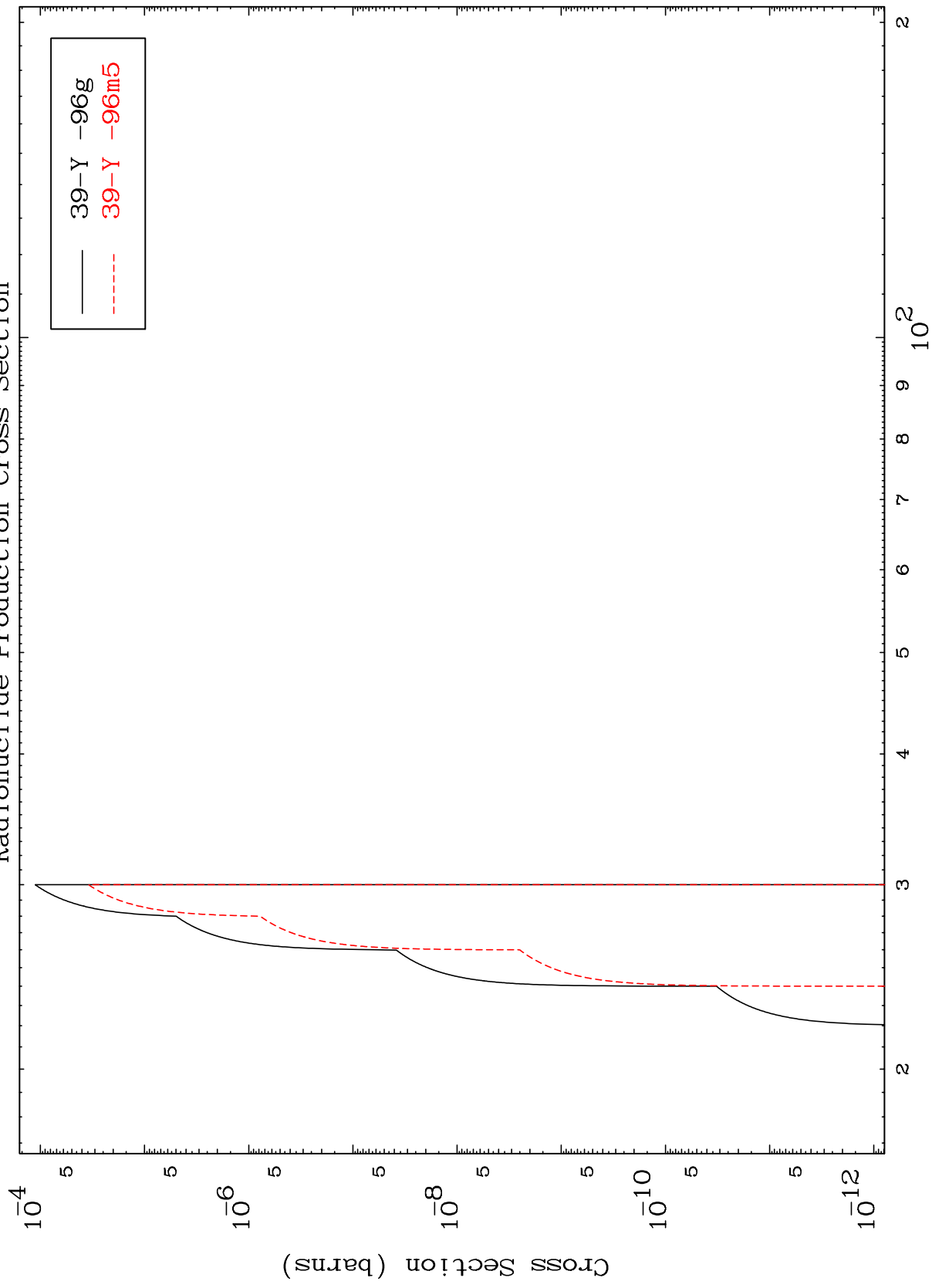
39-Y -99

MAT 3955

$(n,3n) \alpha$

39-Y -99

Radionuclide Production Cross Section



15

Incident Energy (MeV)

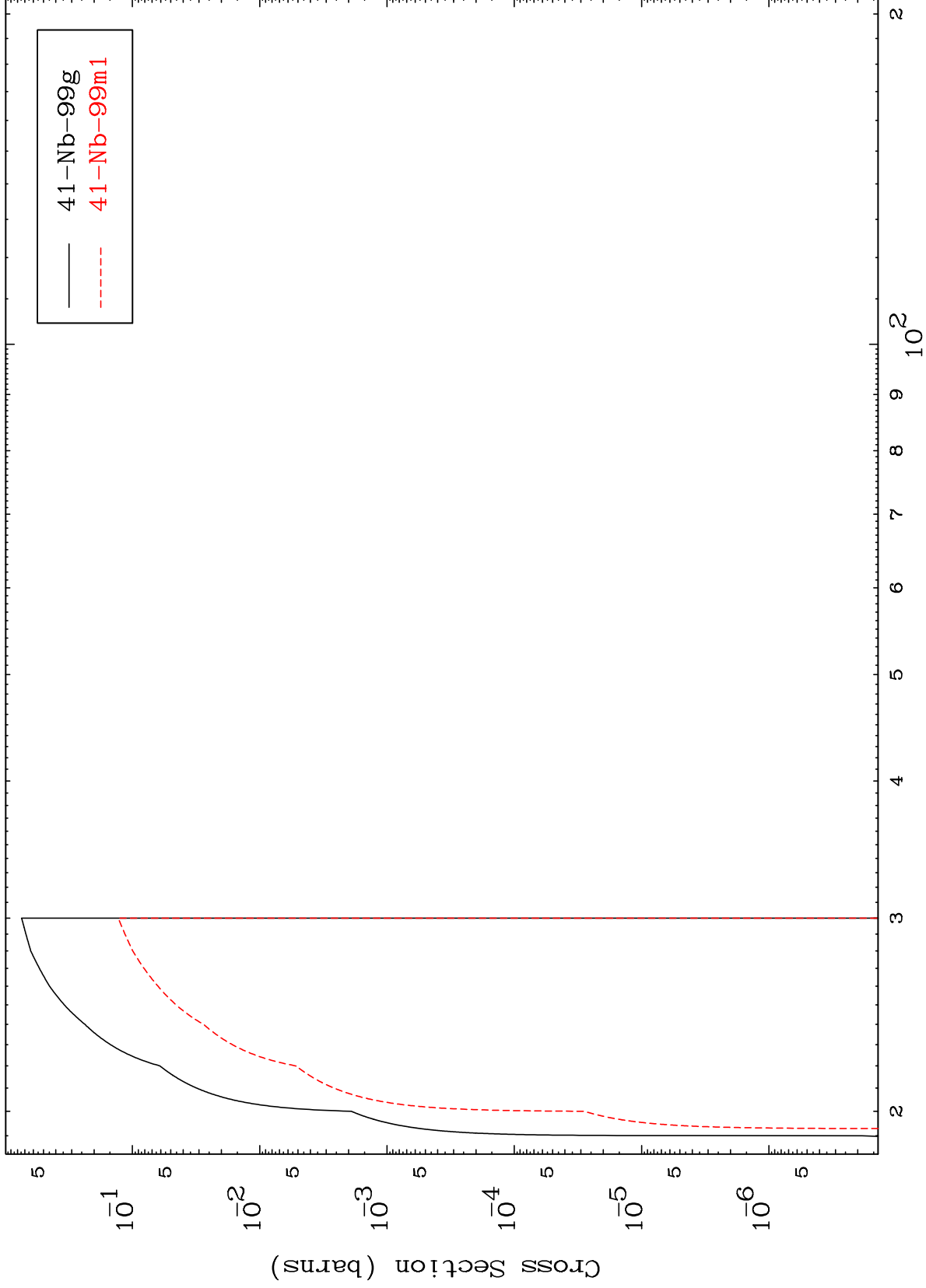
39-Y -99

MAT 3955

(n,4n)

39-Y -99

Radionuclide Production Cross Section



16

Incident Energy (MeV)

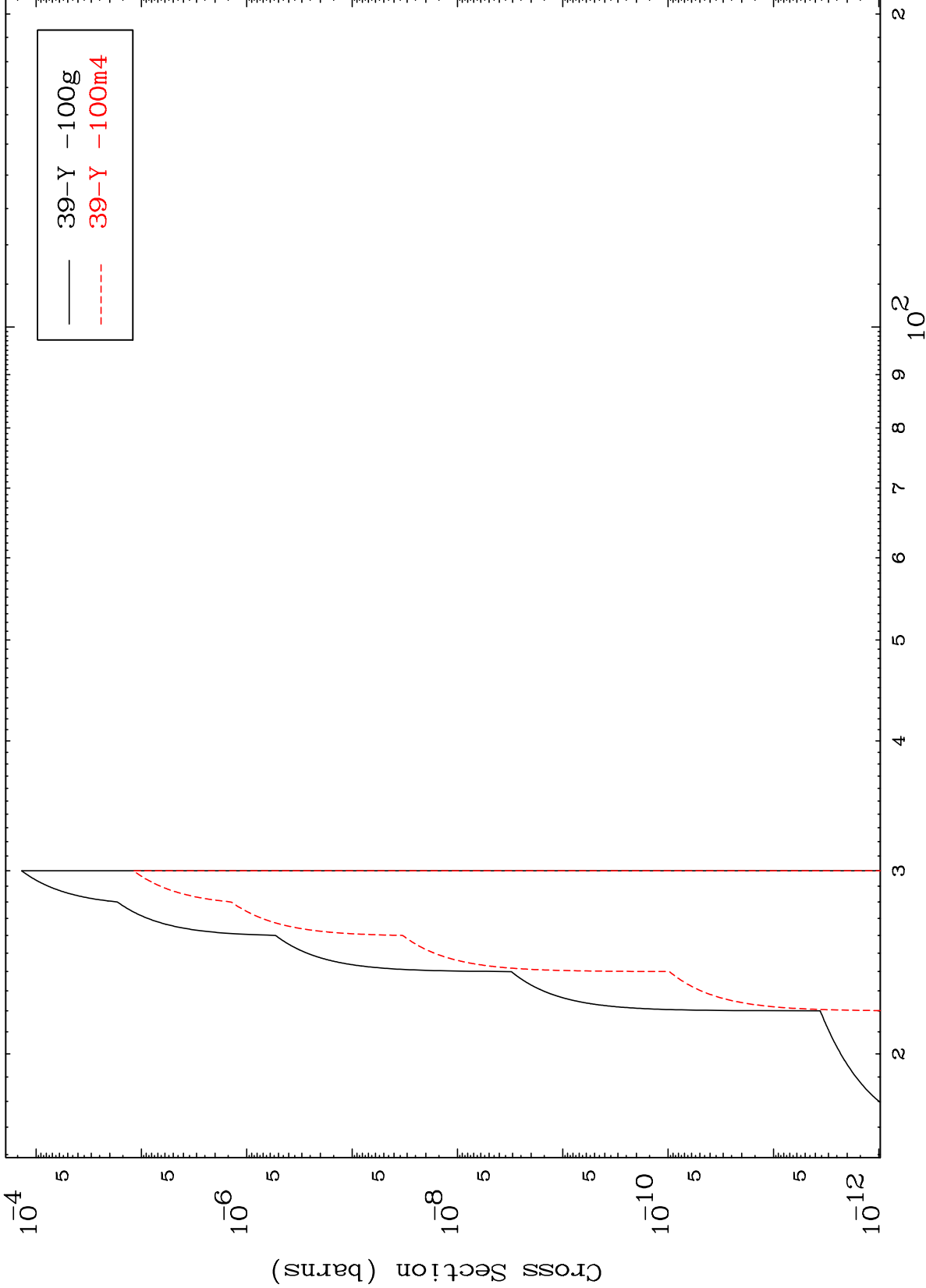
39-Y -99

MAT 3955

(n,He-3)

39-Y -99

Radionuclide Production Cross Section



39-Y -100g  
39-Y -100m4

17

Incident Energy (MeV)

39-Y -99