

Program EVALPLOT  
(Version 2021-1)

by

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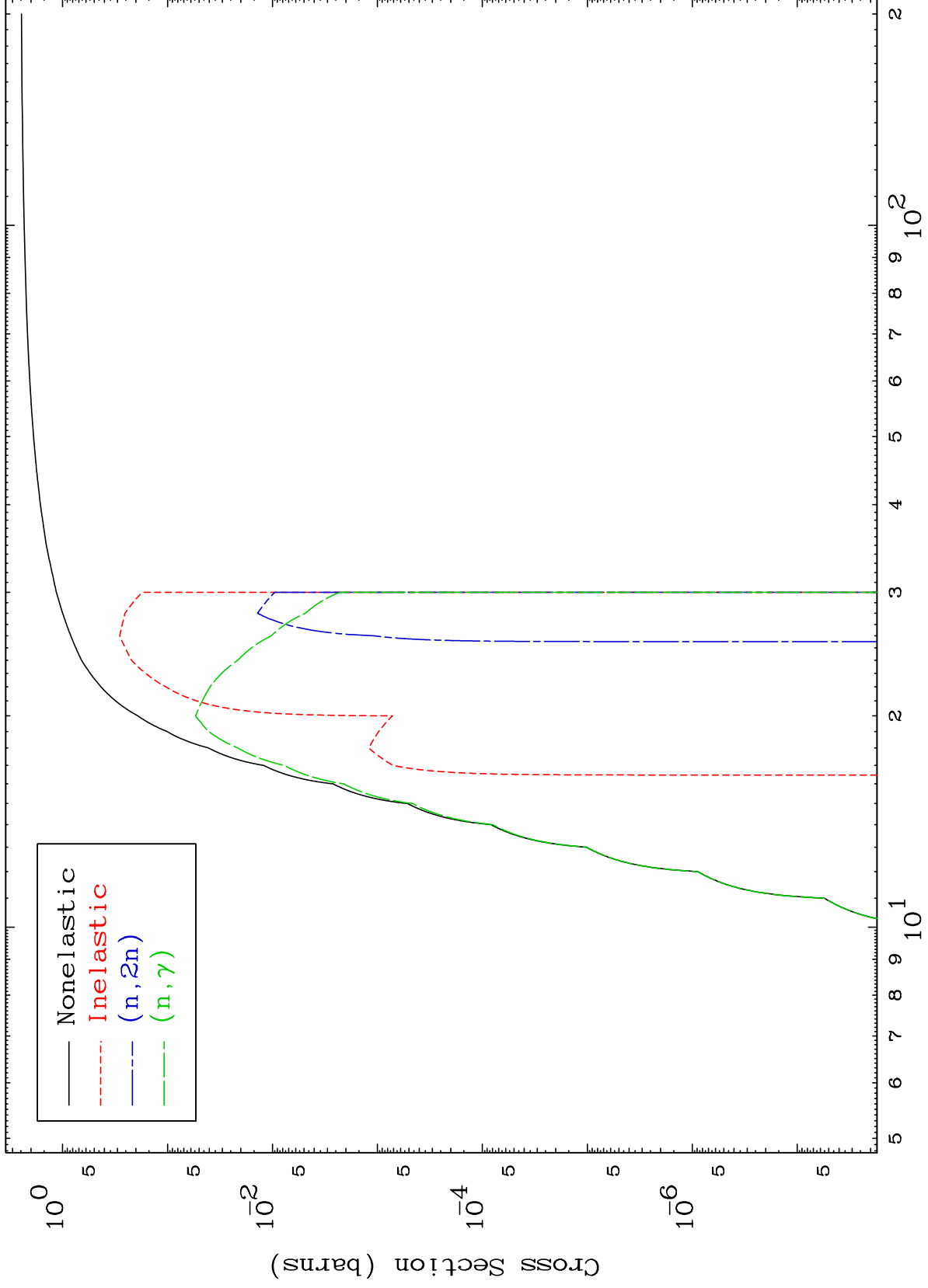
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 6989

0 Kelvin Cross Sections

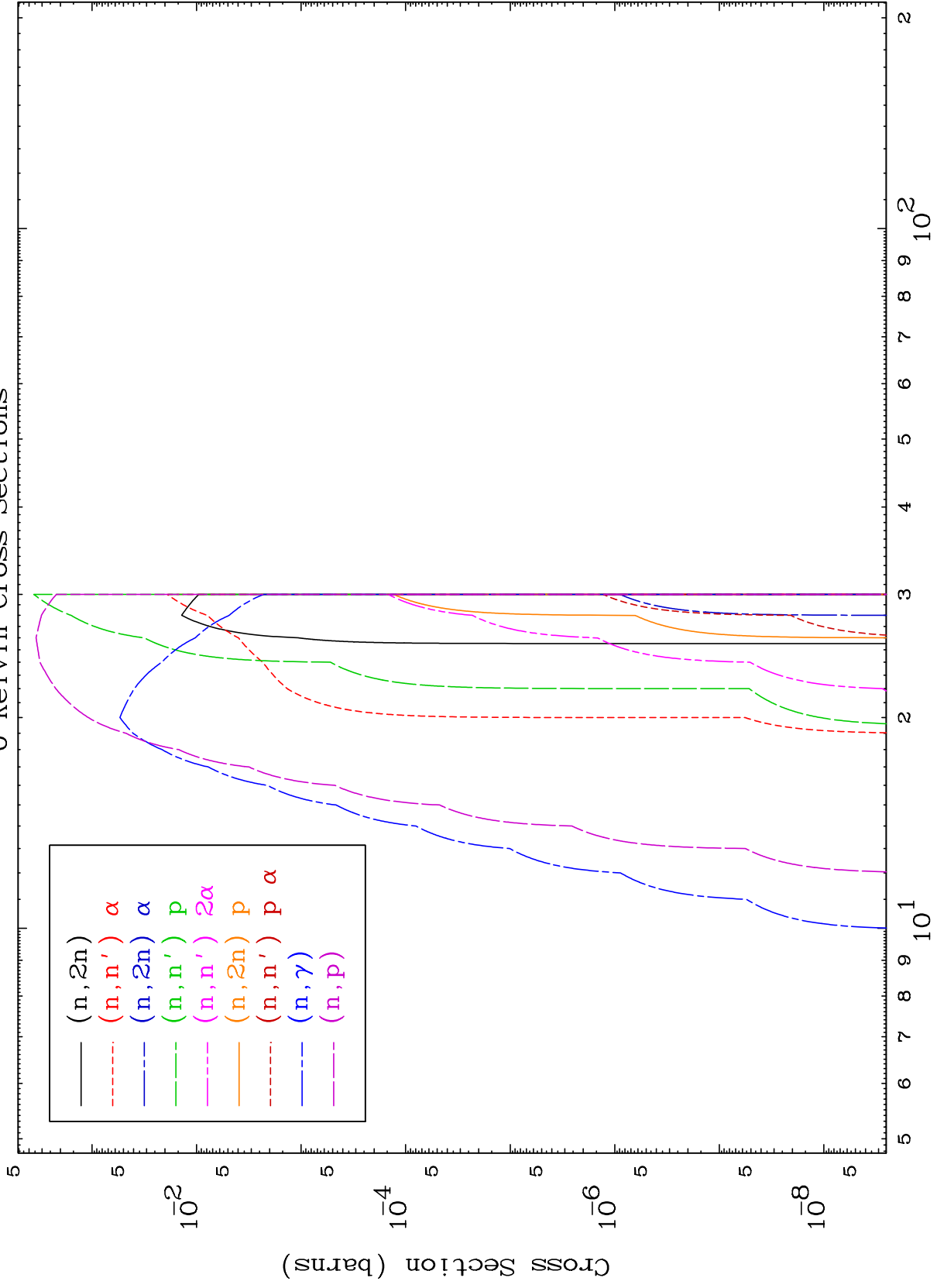
70-Yb-156



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Incident Energy (MeV)

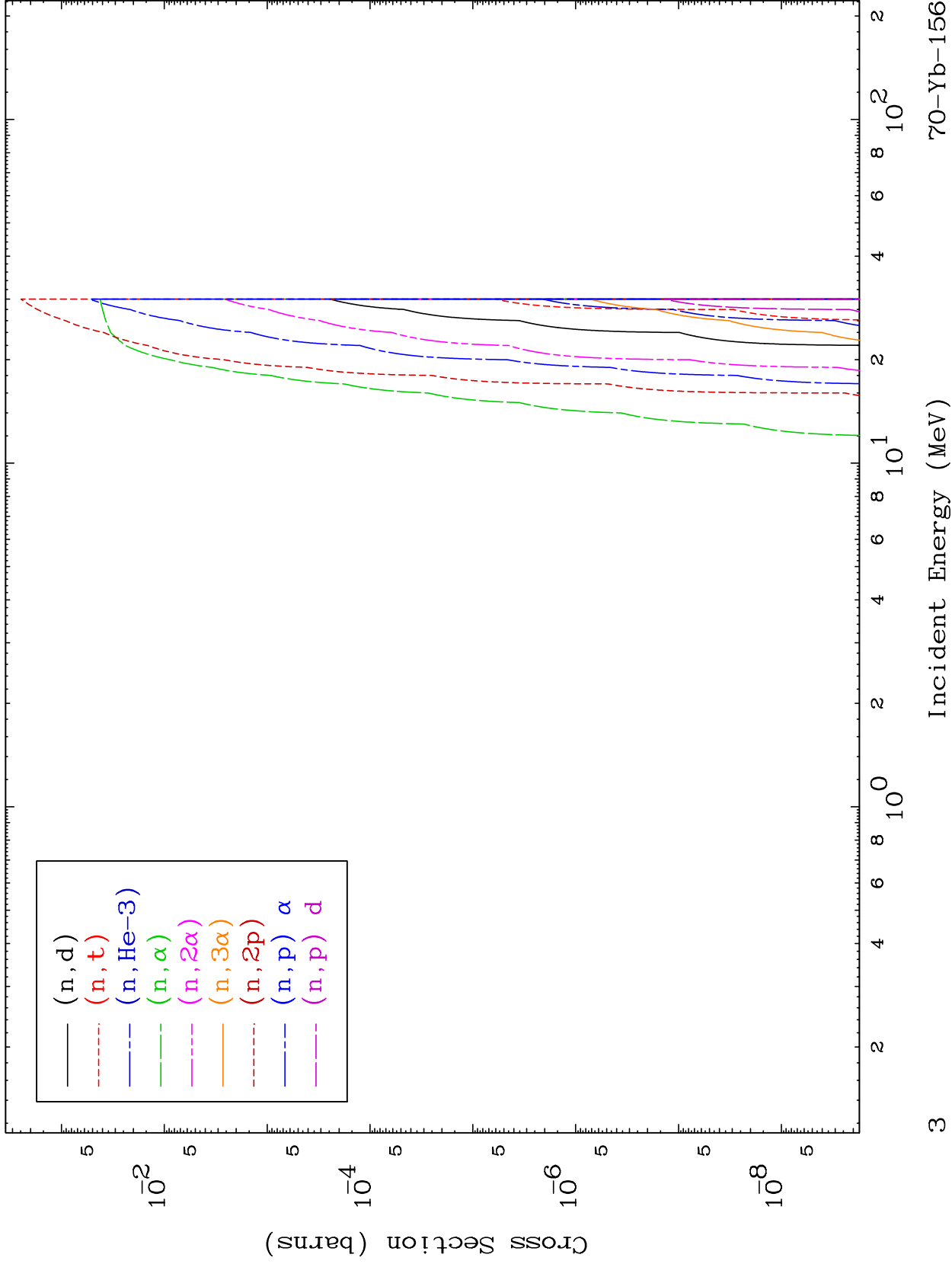
70-Yb-156

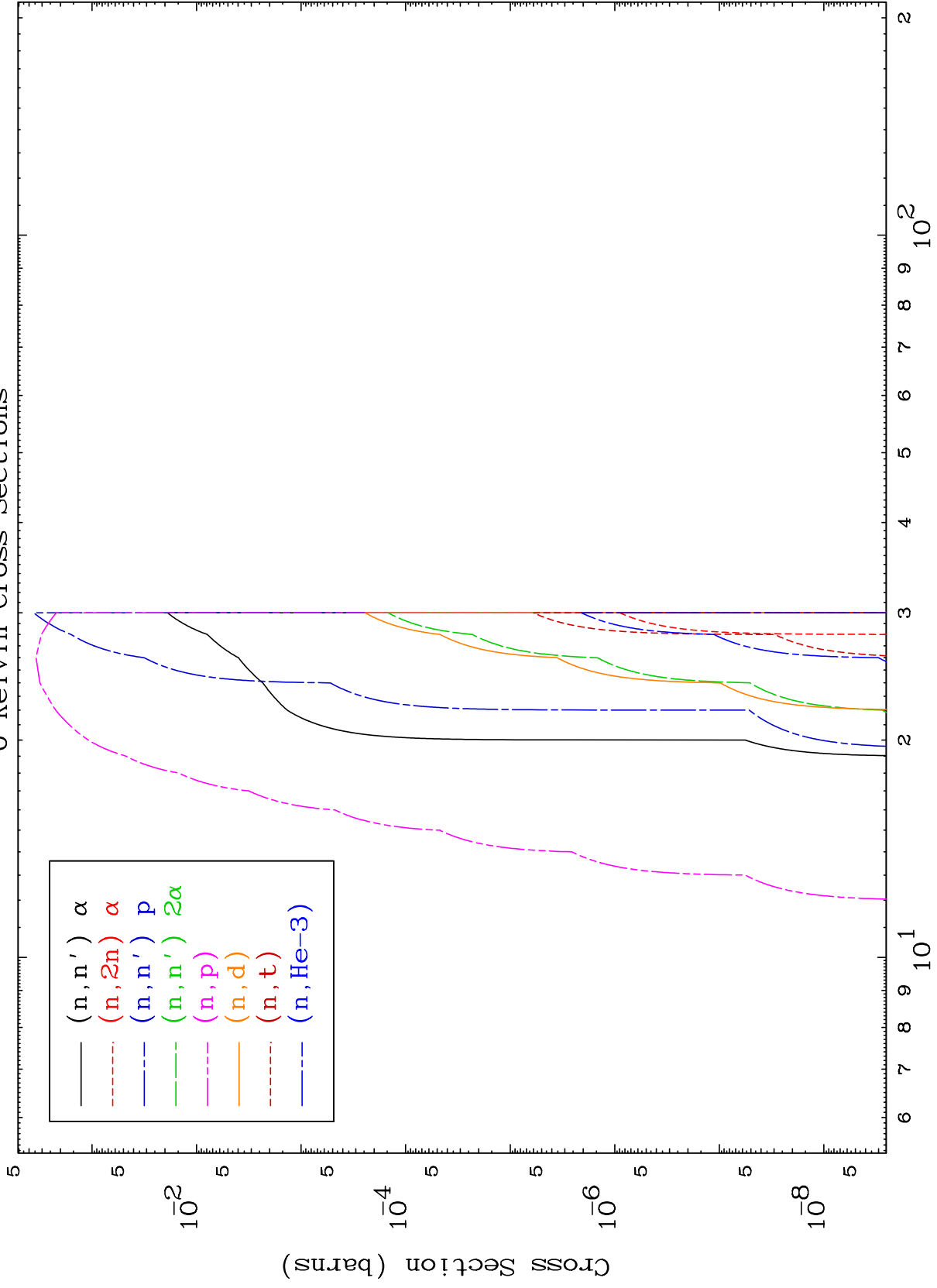


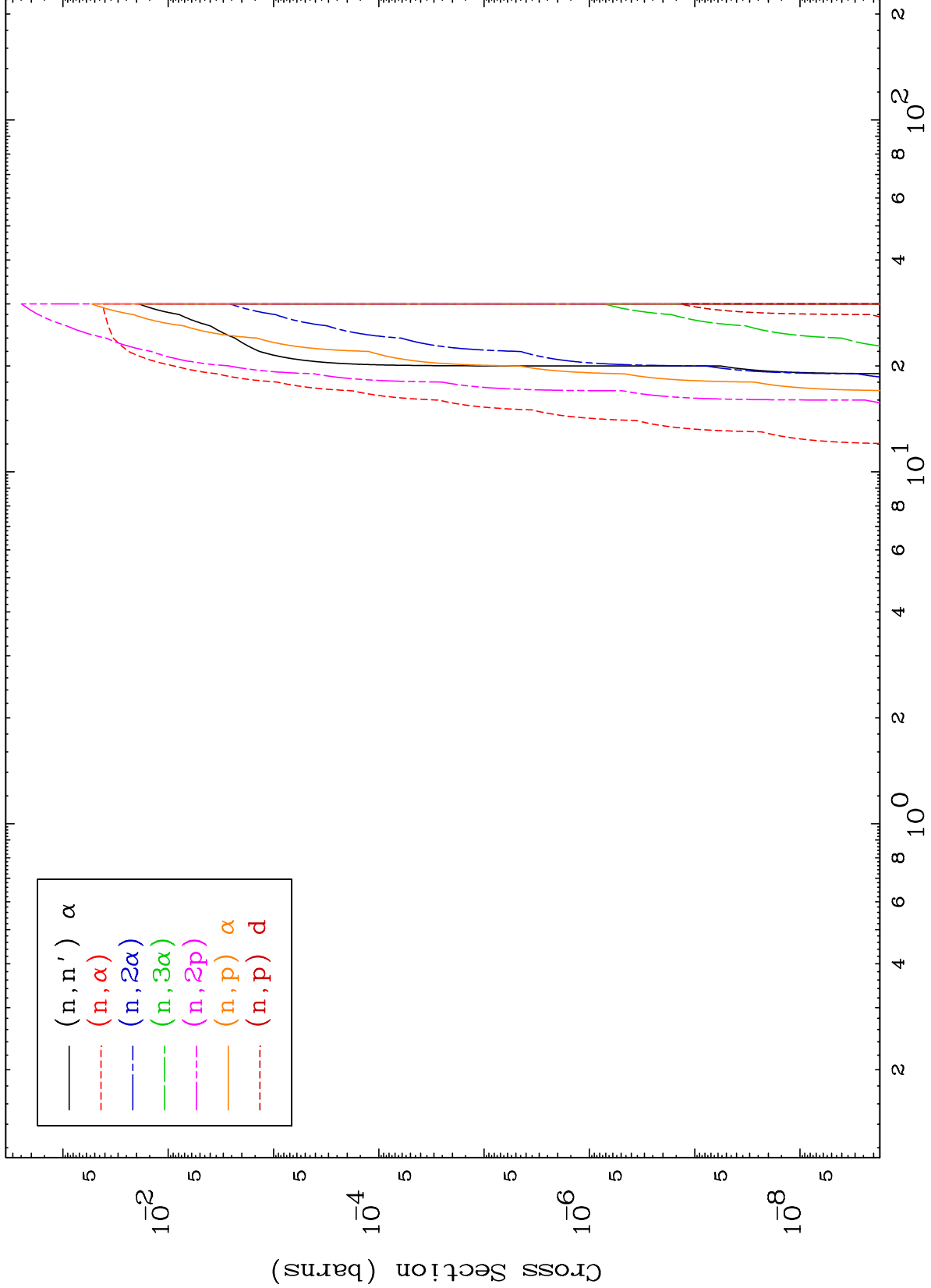
MAT 6989

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

70-Yb-156



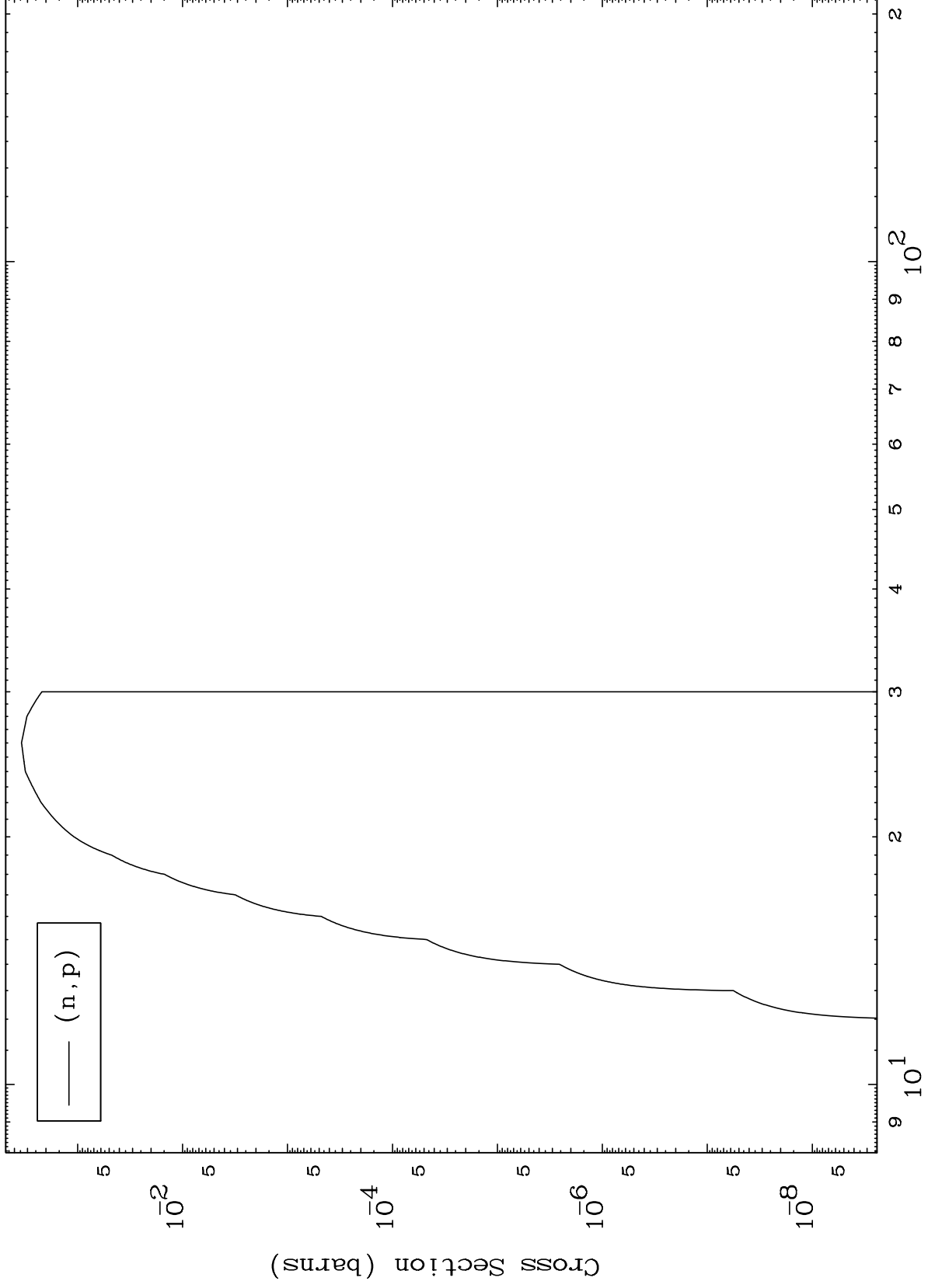




MAT 6989

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

70-Yb-156



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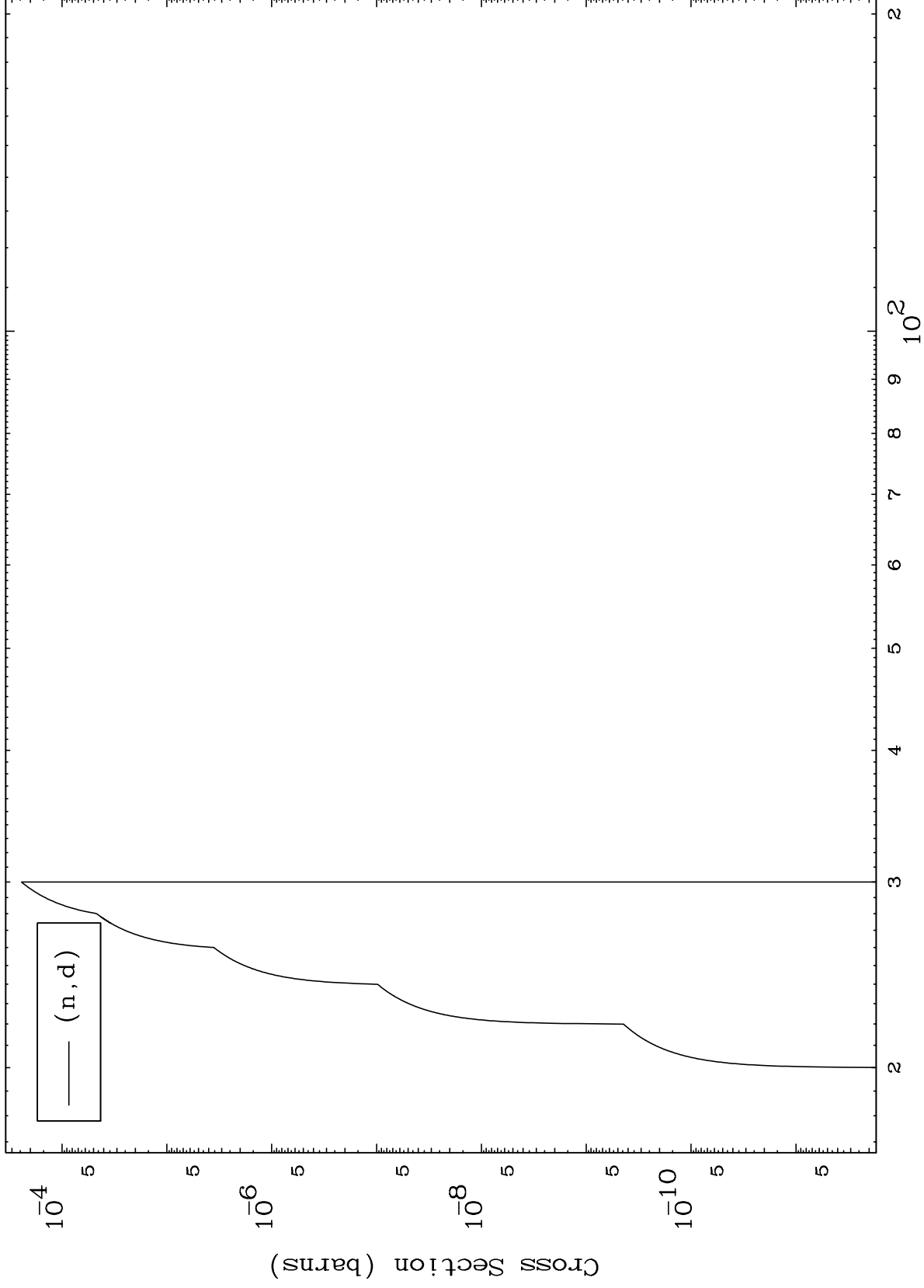
Incident Energy (MeV)

70-Yb-156

MAT 6989

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

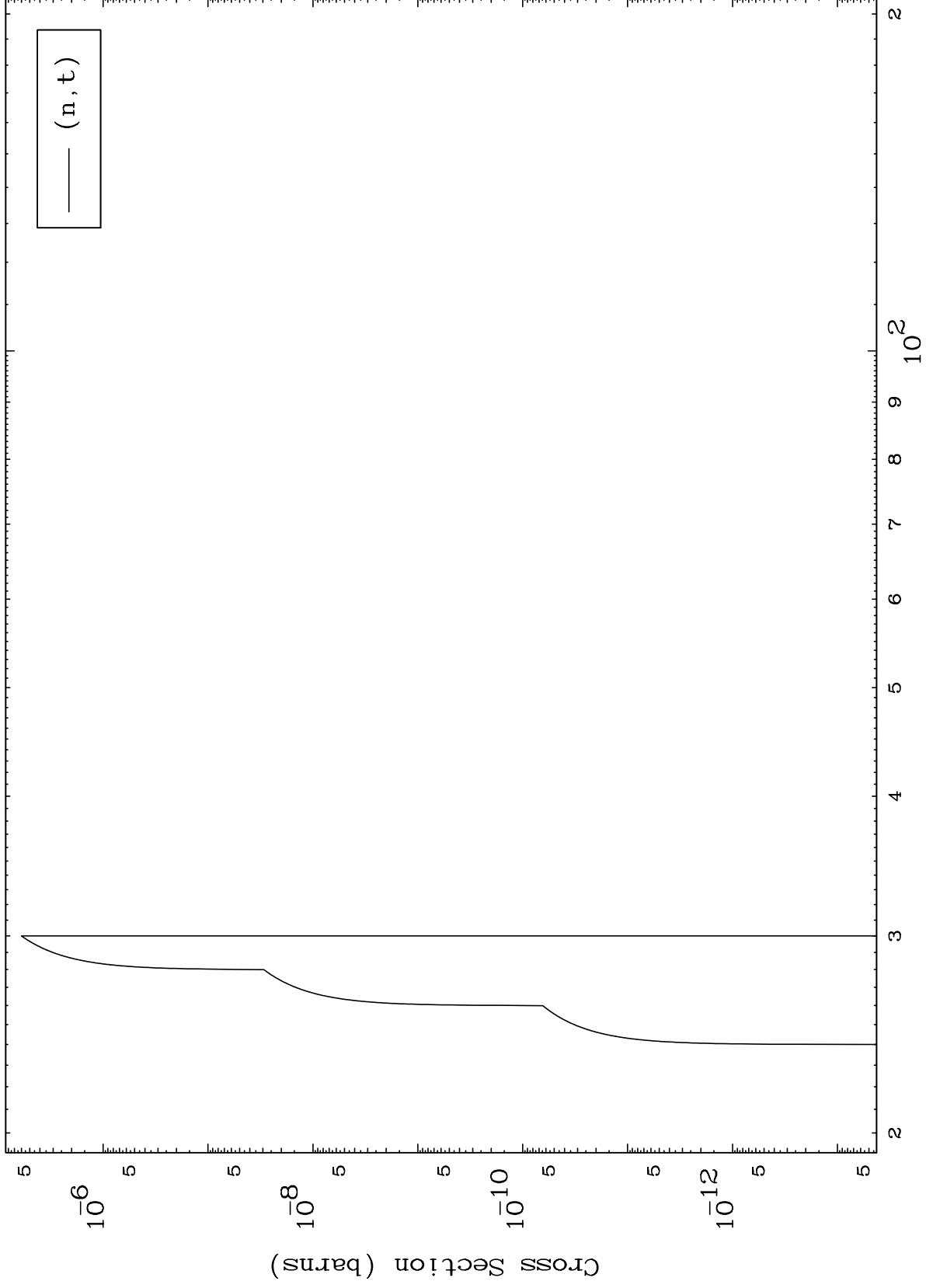
70-Yb-156

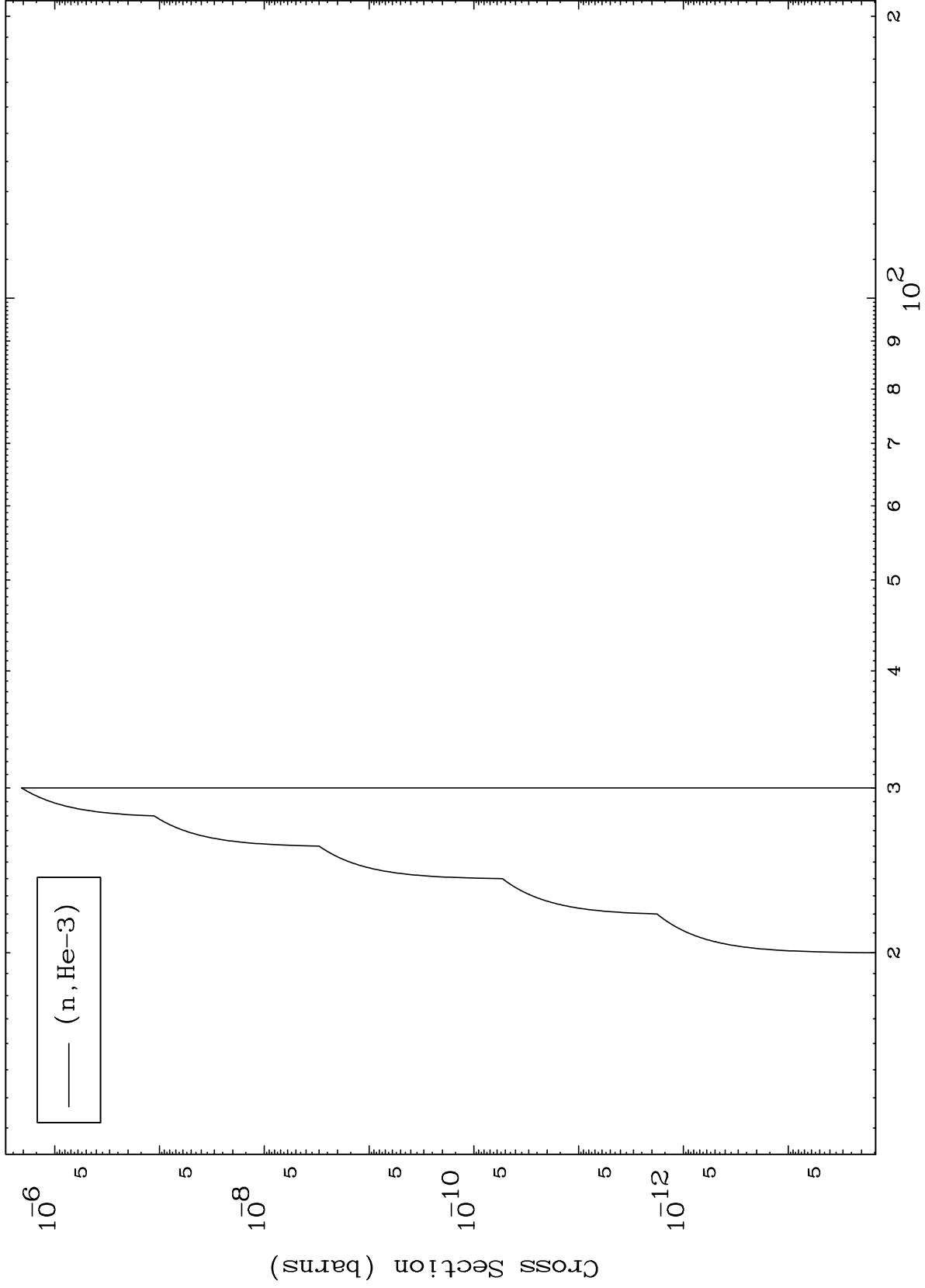


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Incident Energy (MeV)

70-Yb-156



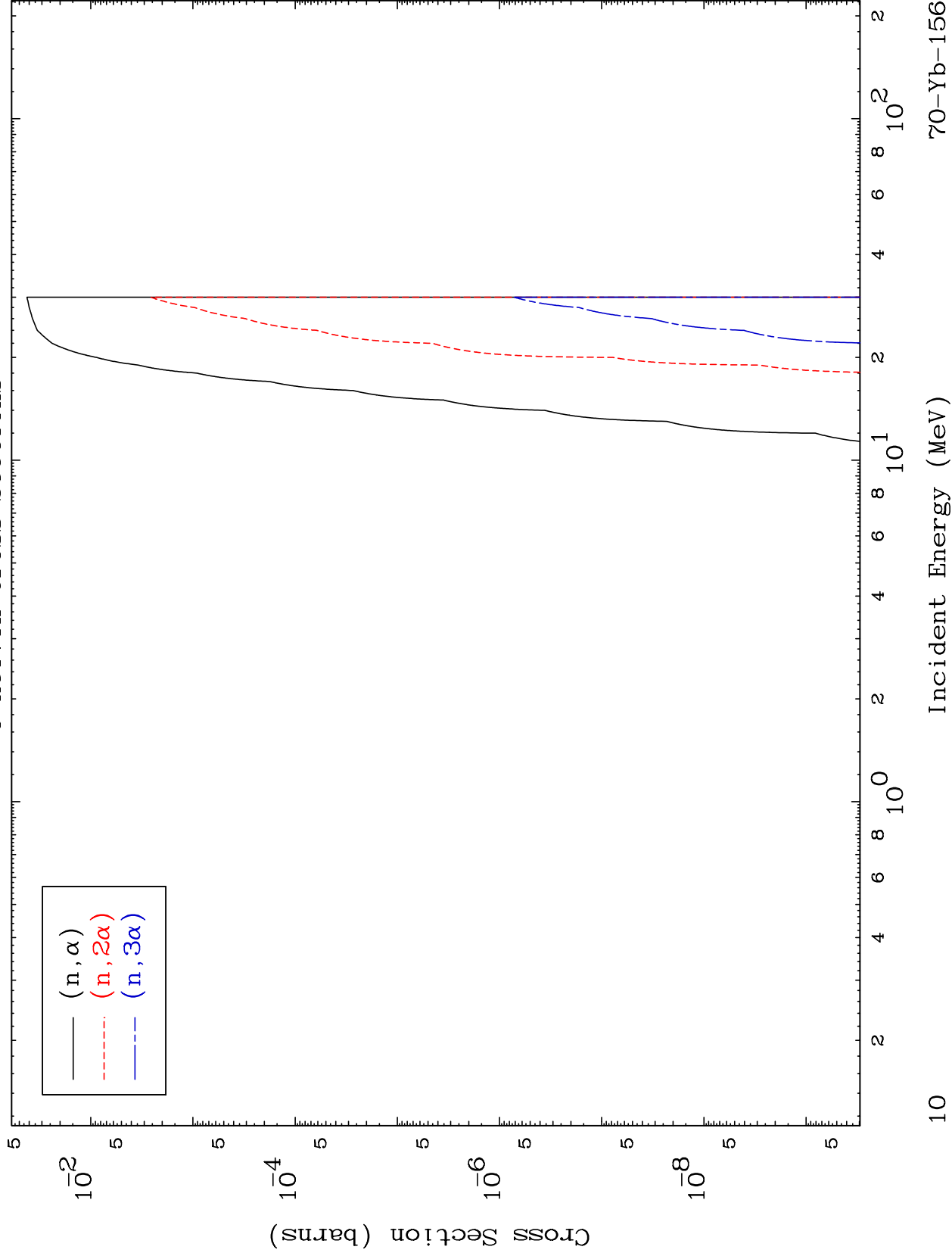


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( $\alpha, \alpha$ ) Levels

70-Yb-156

0 Kelvin Cross Sections



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Incident Energy (MeV)

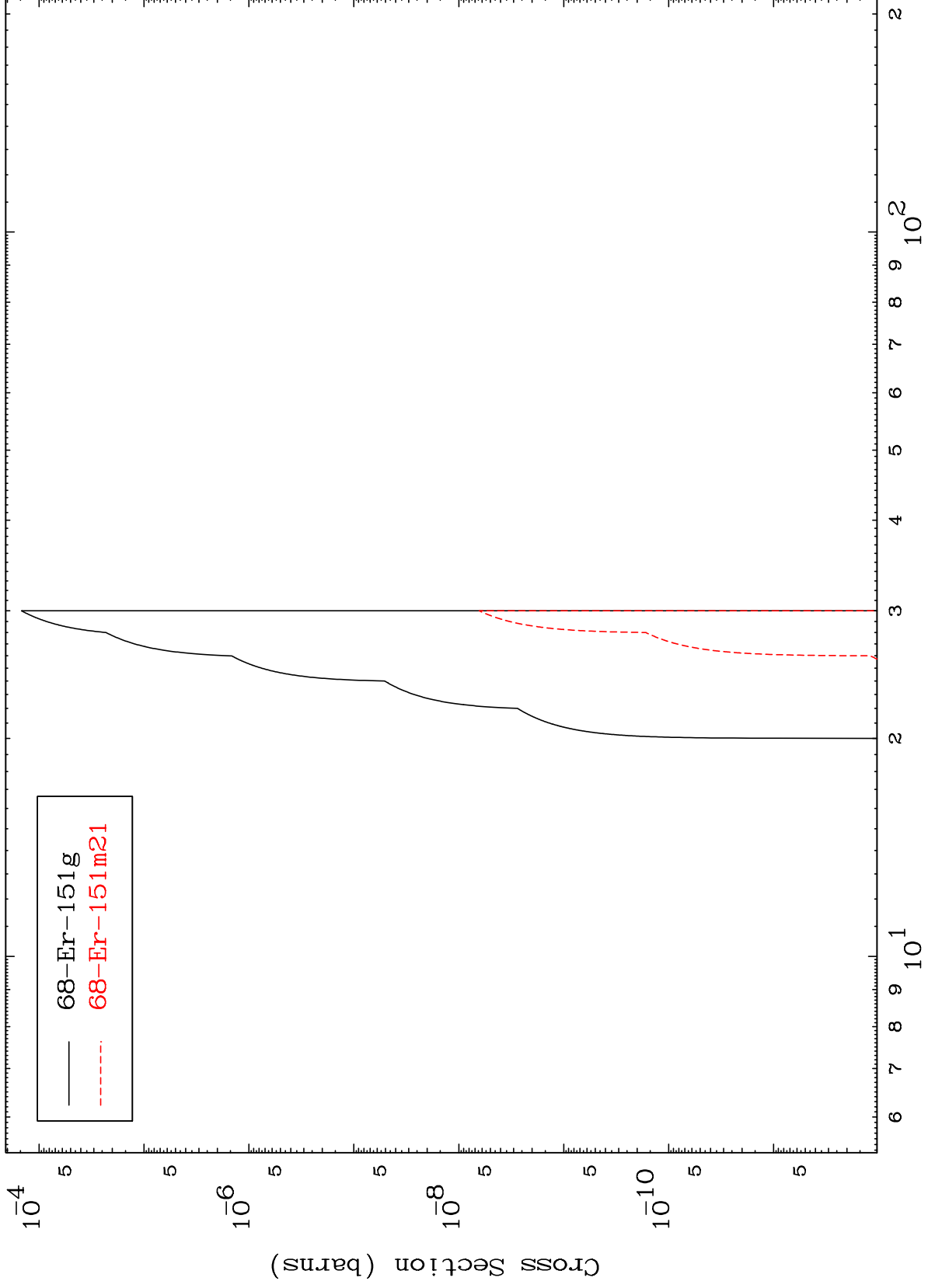
70-Yb-156

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(n,n') 2 $\alpha$

70-Yb-156

Radionuclide Production Cross Section



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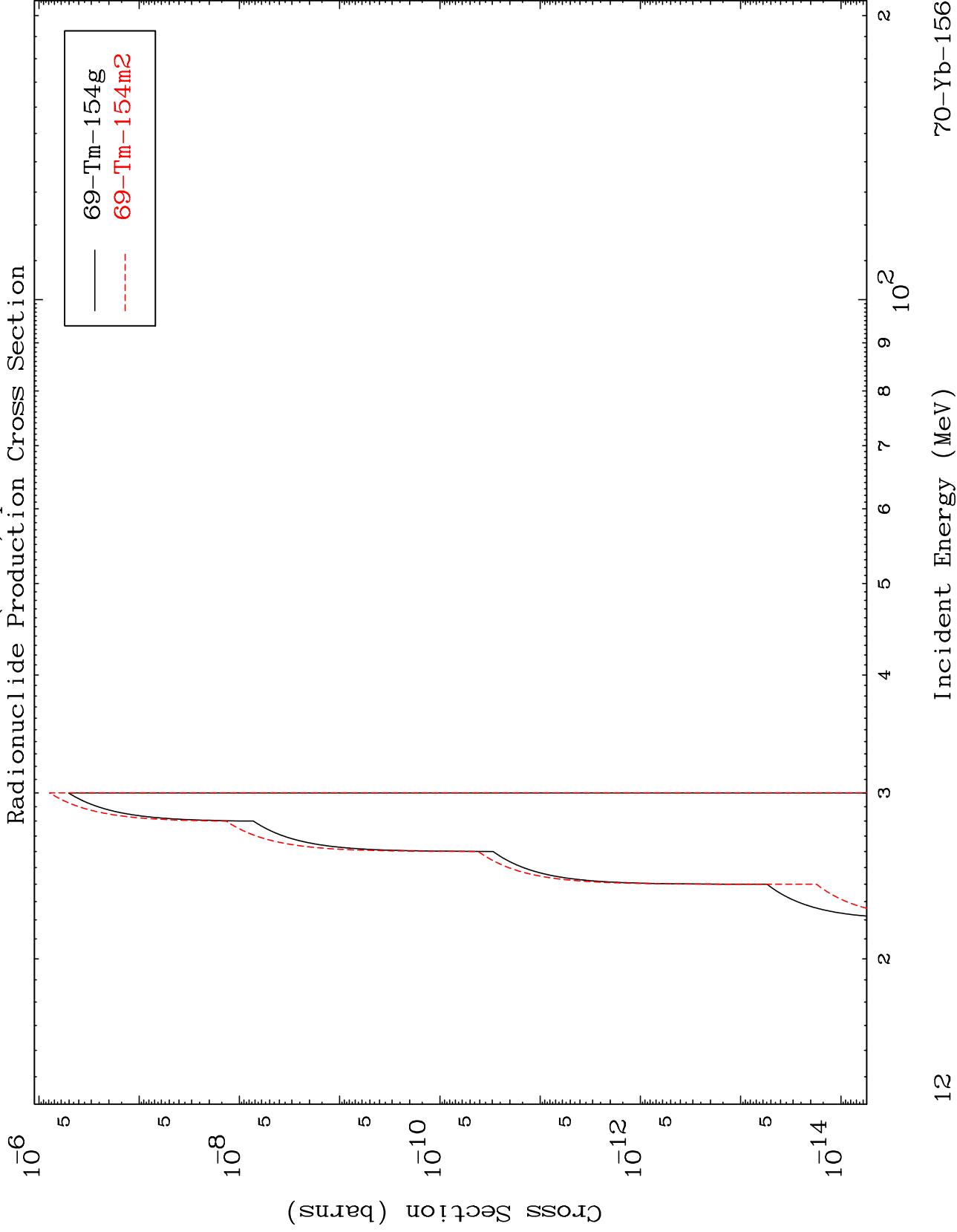
Incident Energy (MeV)

70-Yb-156

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(n,n') p  $\alpha$

70-Yb-156



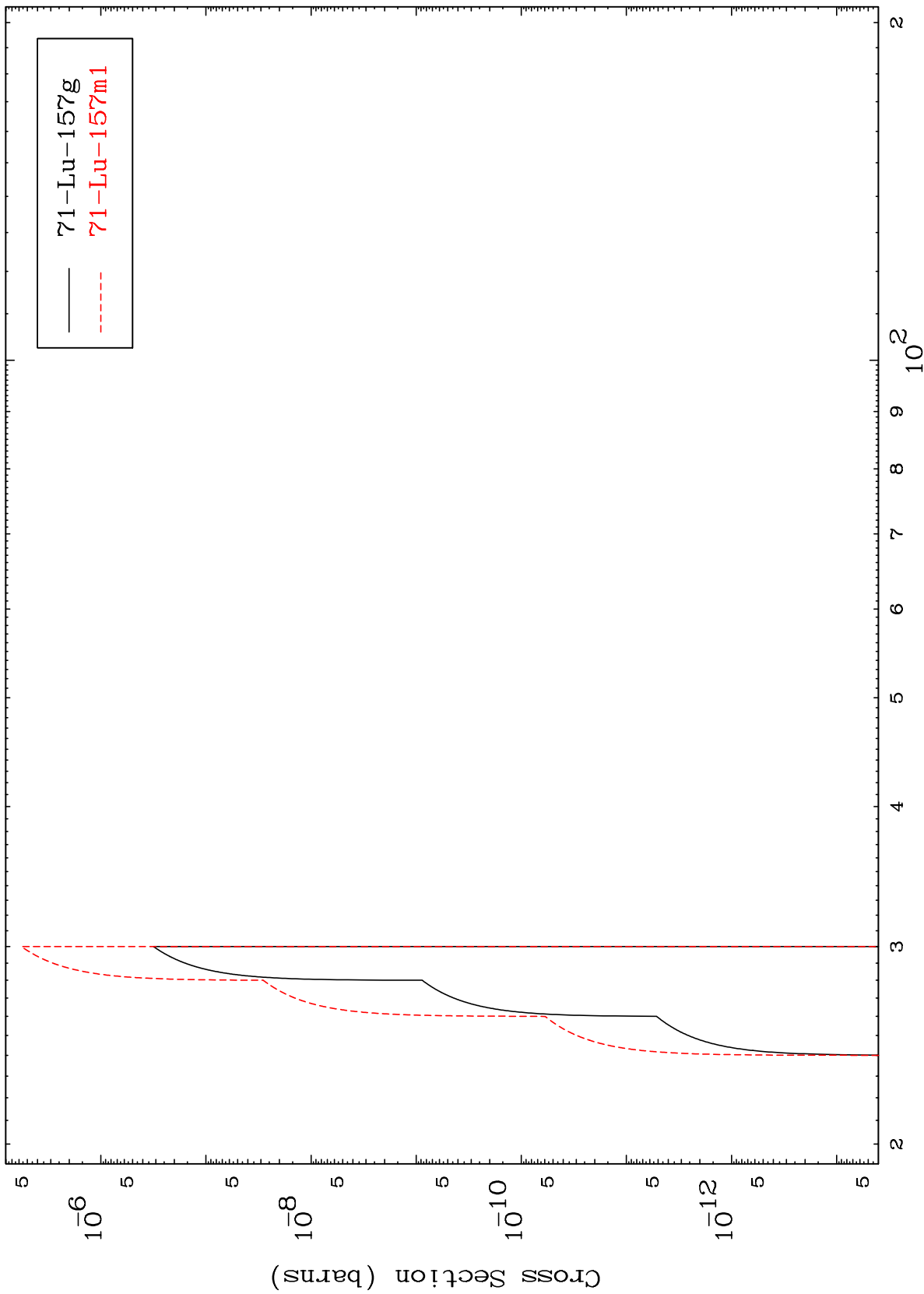
12

70-Yb-156

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70-Yb-156

(n,t)  
Radionuclide Production Cross Section



70-Yb-156

Incident Energy (MeV)

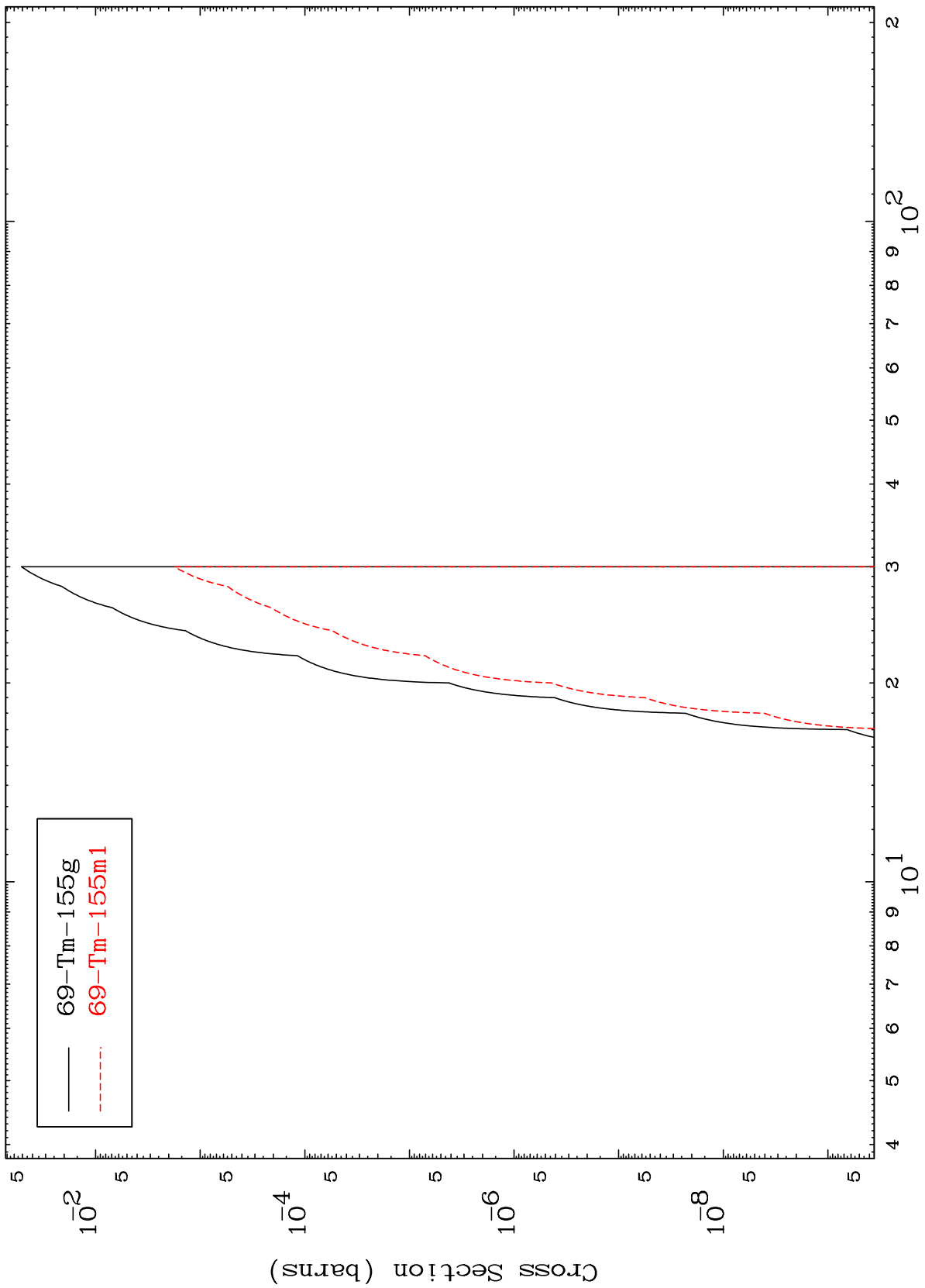
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(n,p)  $\alpha$

70-Yb-156

Radionuclide Production Cross Section



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Incident Energy (MeV)

70-Yb-156