

Program EVALPLOT  
(Version 2021-1)

by

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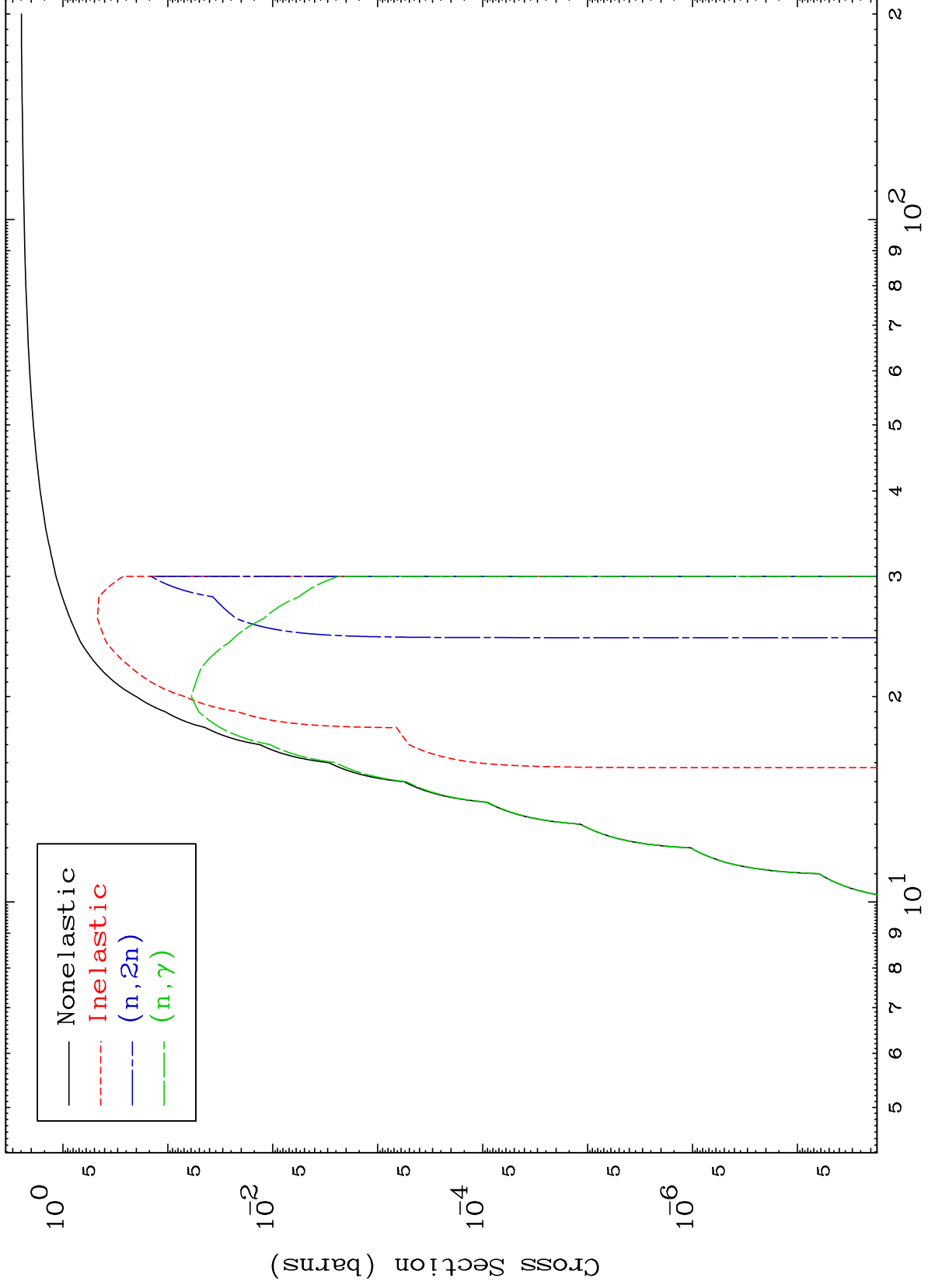
Press Mouse Button to Start

MAT 6995

0 Kelvin Cross Sections

$\alpha$  Major

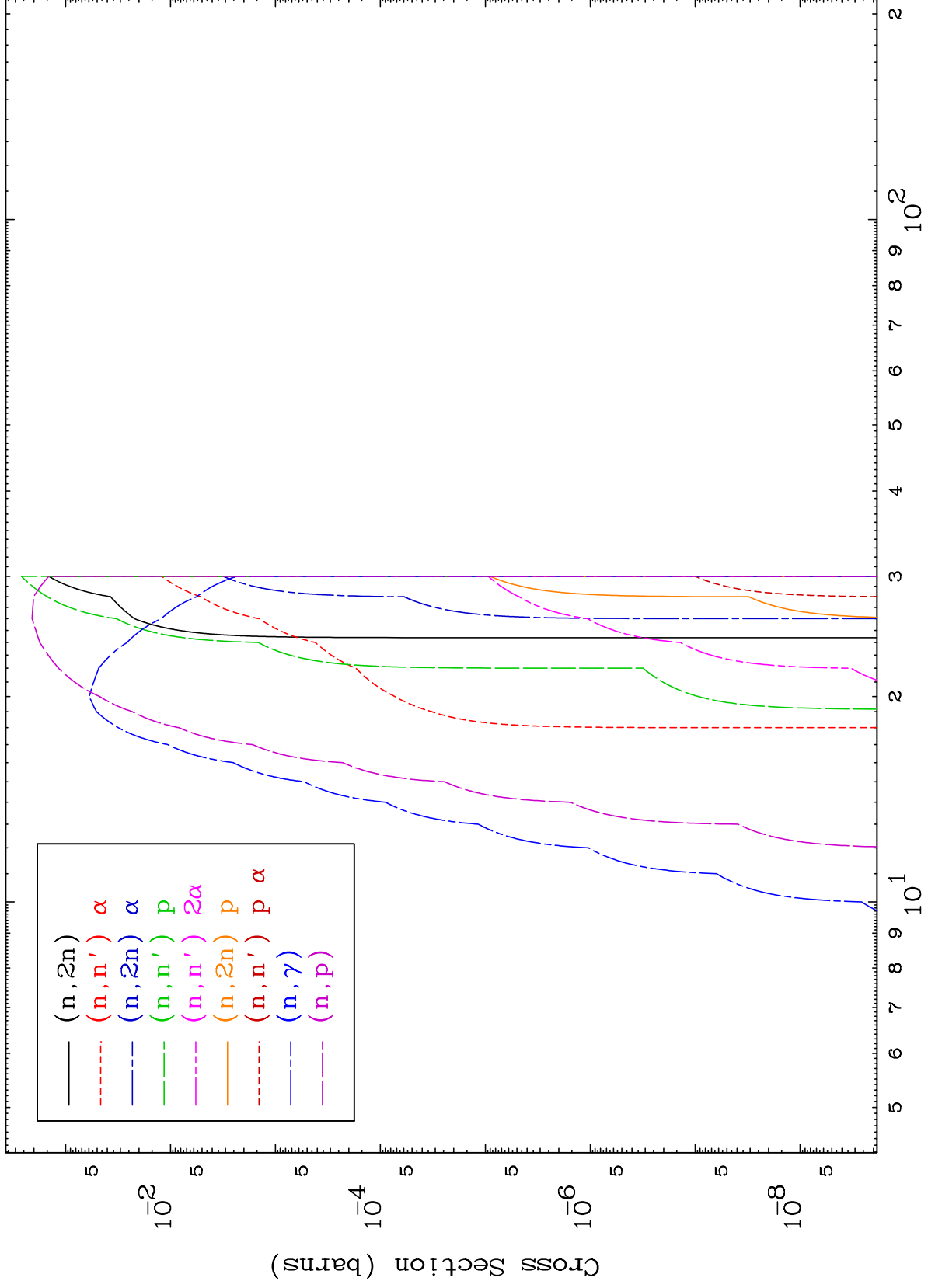
70-Yb-158



1

Incident Energy (MeV)

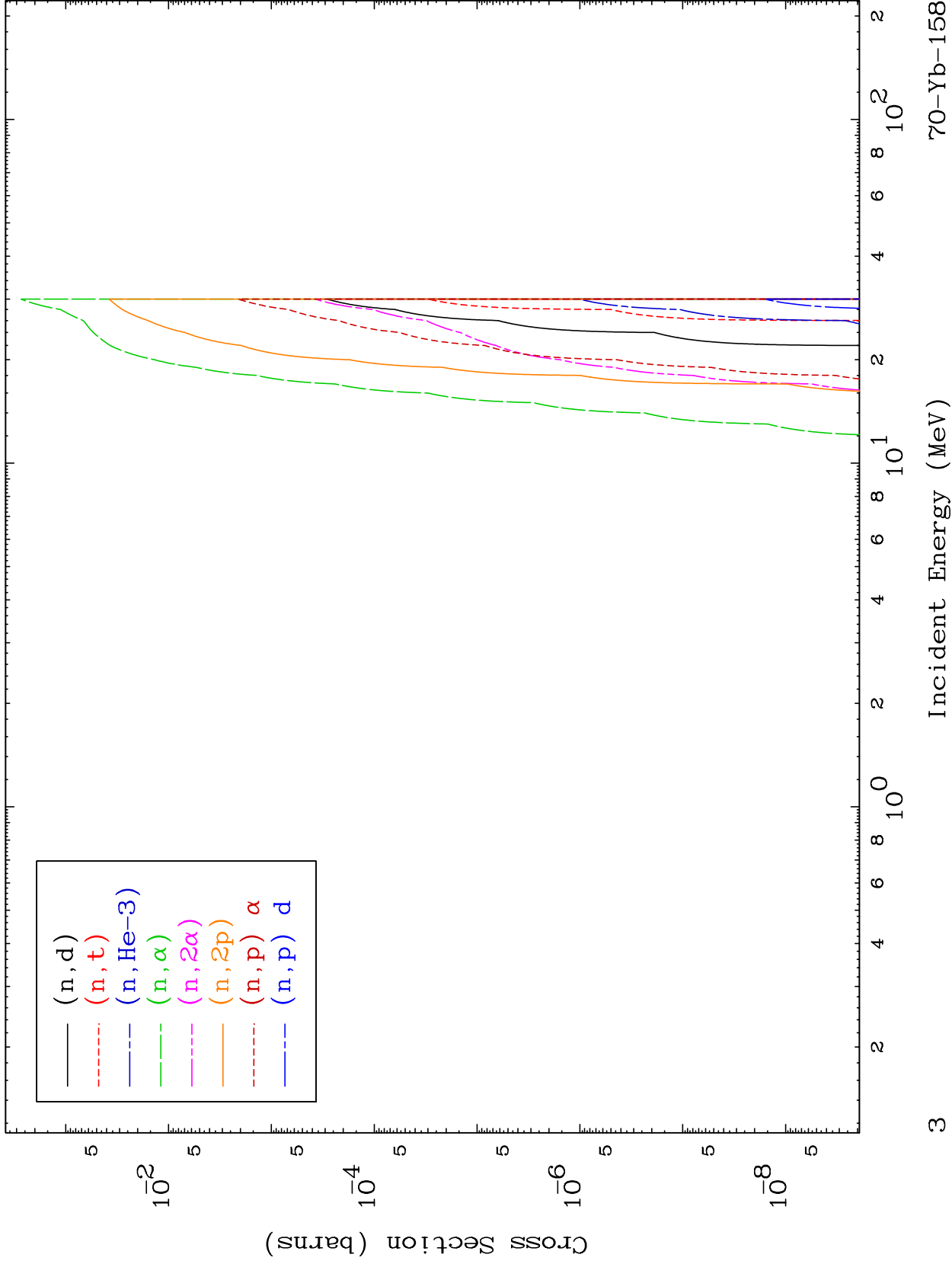
70-Yb-158

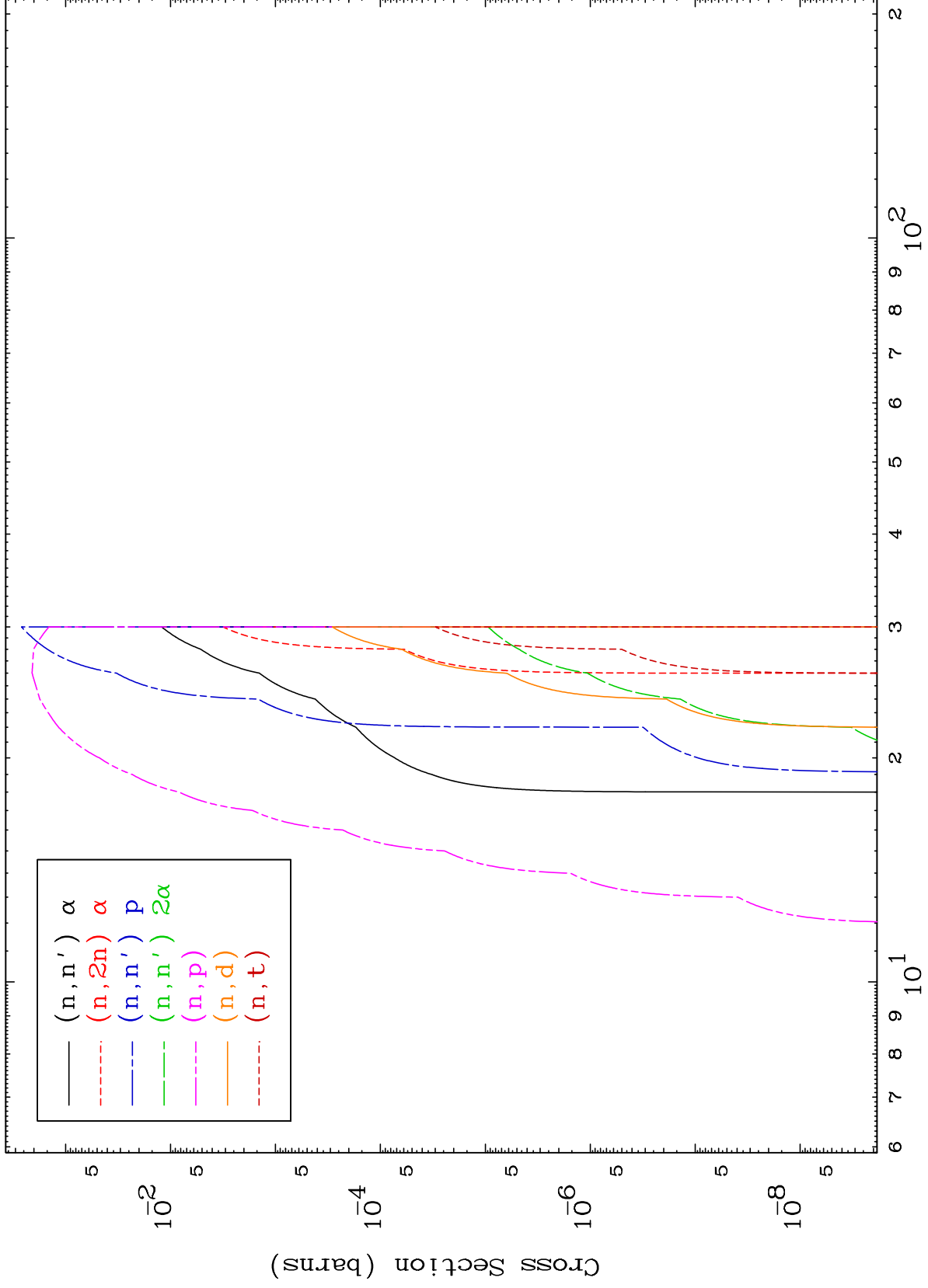


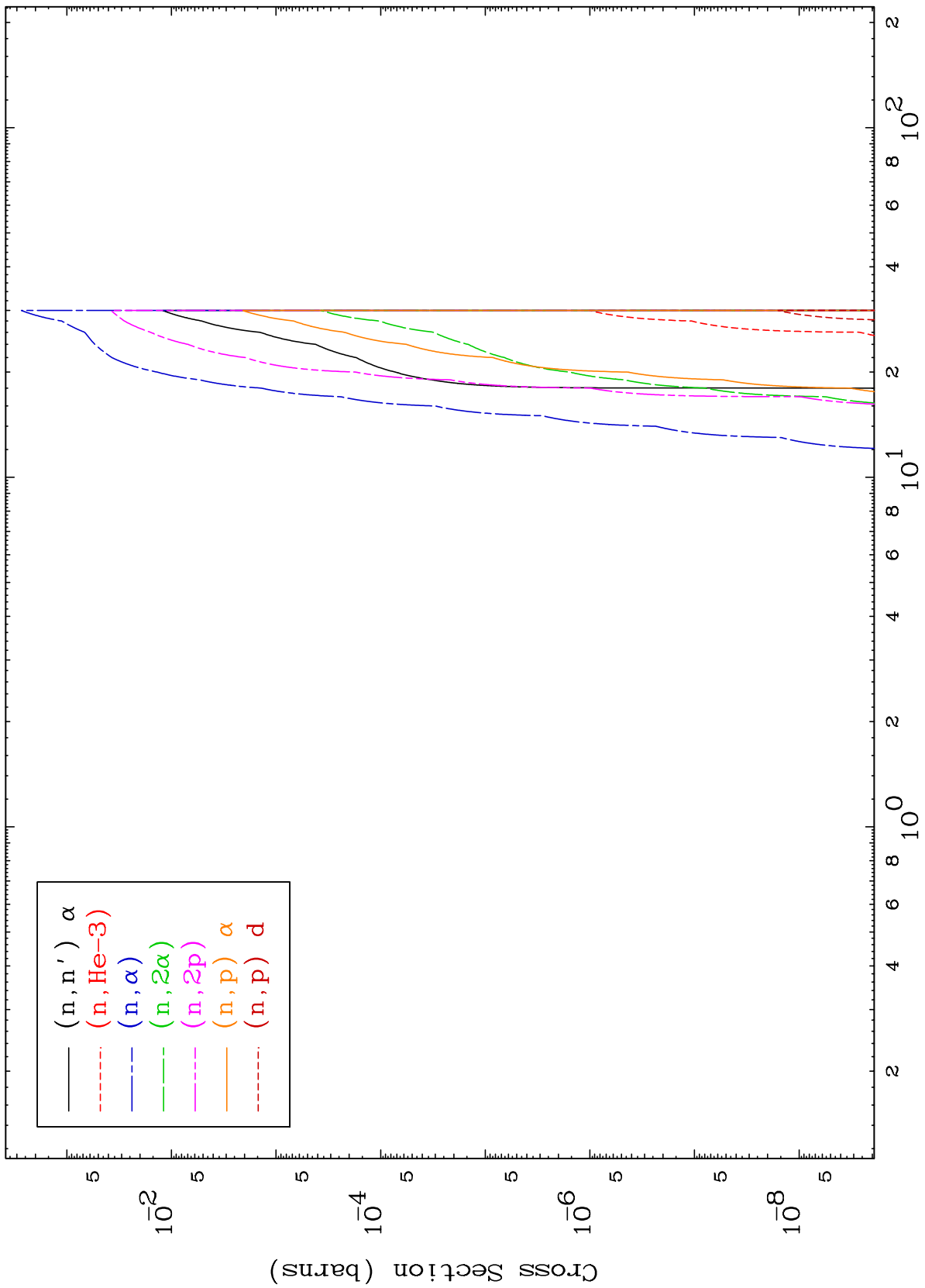
MAT 6995

$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

70-Yb-158





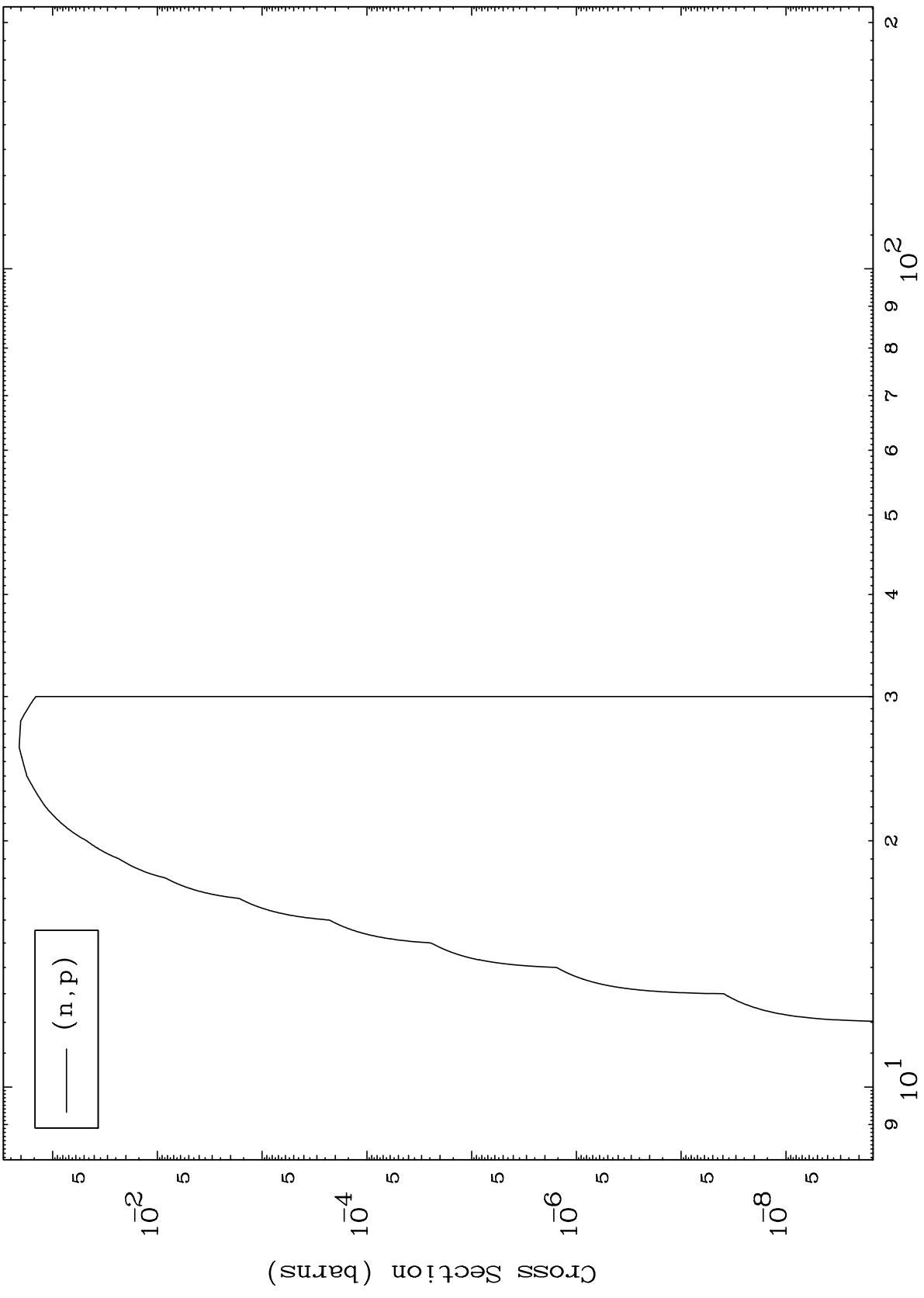


MAT 6995

( $\alpha, p$ ) Levels

70-Yb-158

0 Kelvin Cross Sections



6

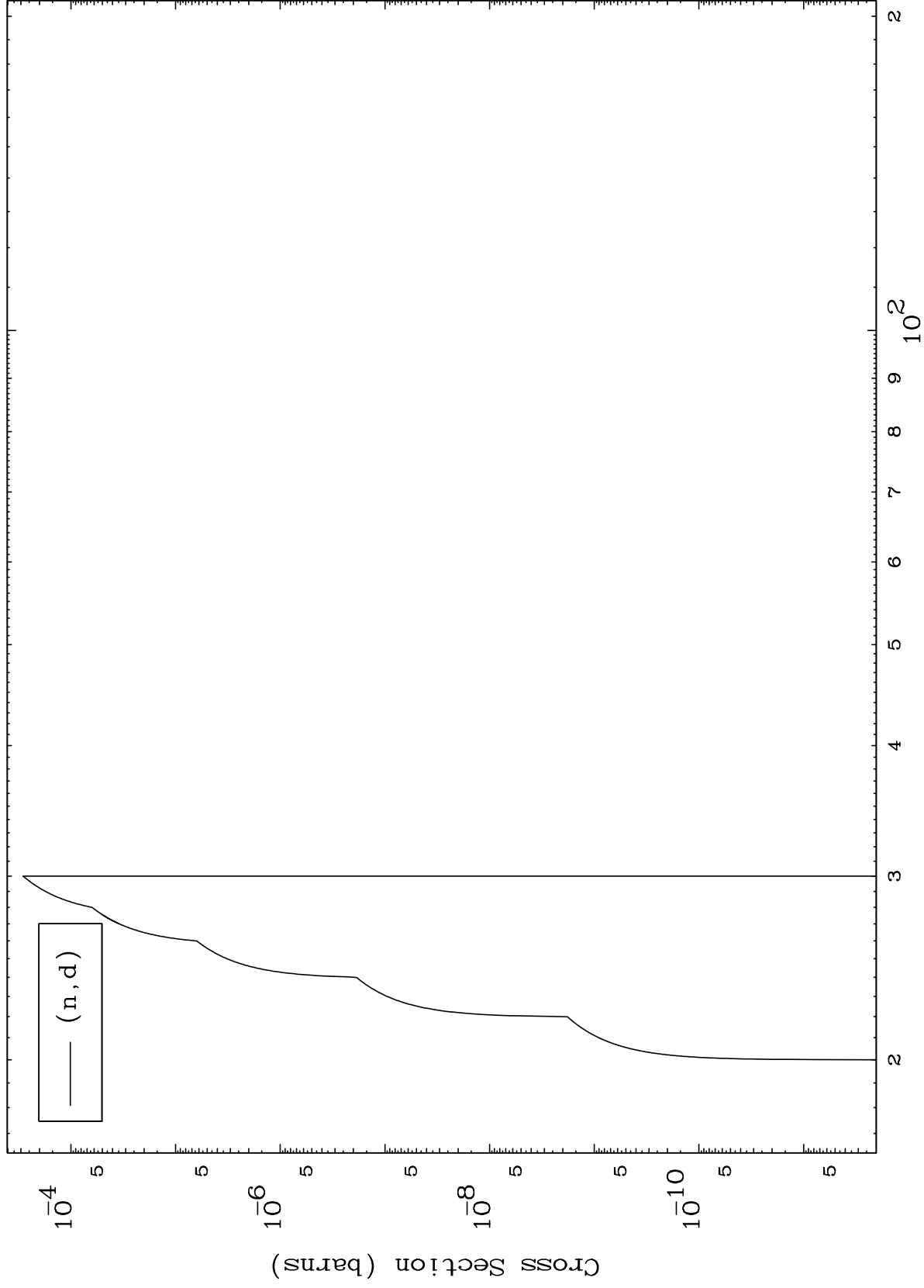
Incident Energy (MeV)

70-Yb-158

MAT 6995

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

70-Yb-158



7

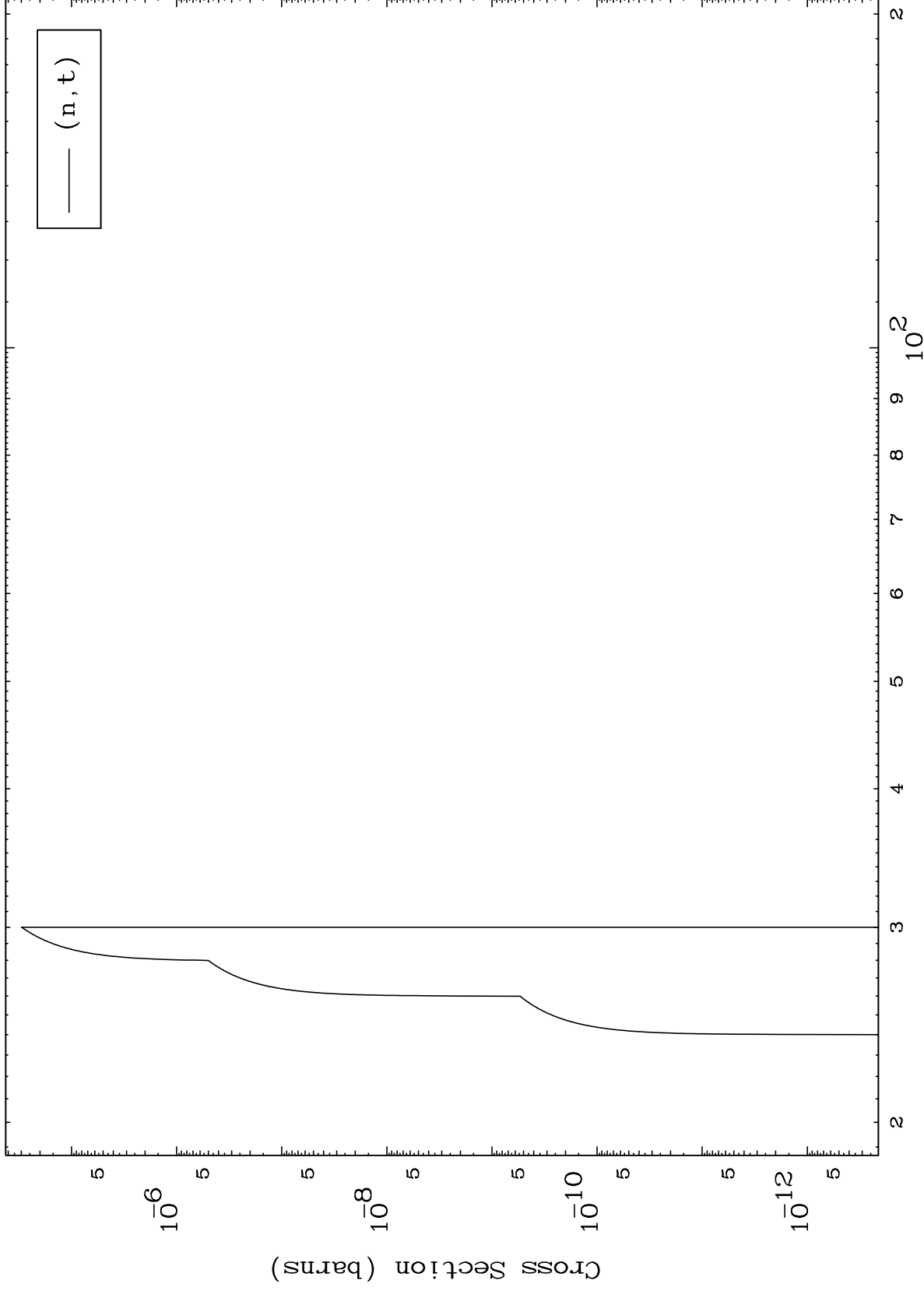
Incident Energy (MeV)

70-Yb-158

MAT 6995

( $\alpha, t$ ) Levels  
0 Kelvin Cross Sections

70-Yb-158



8

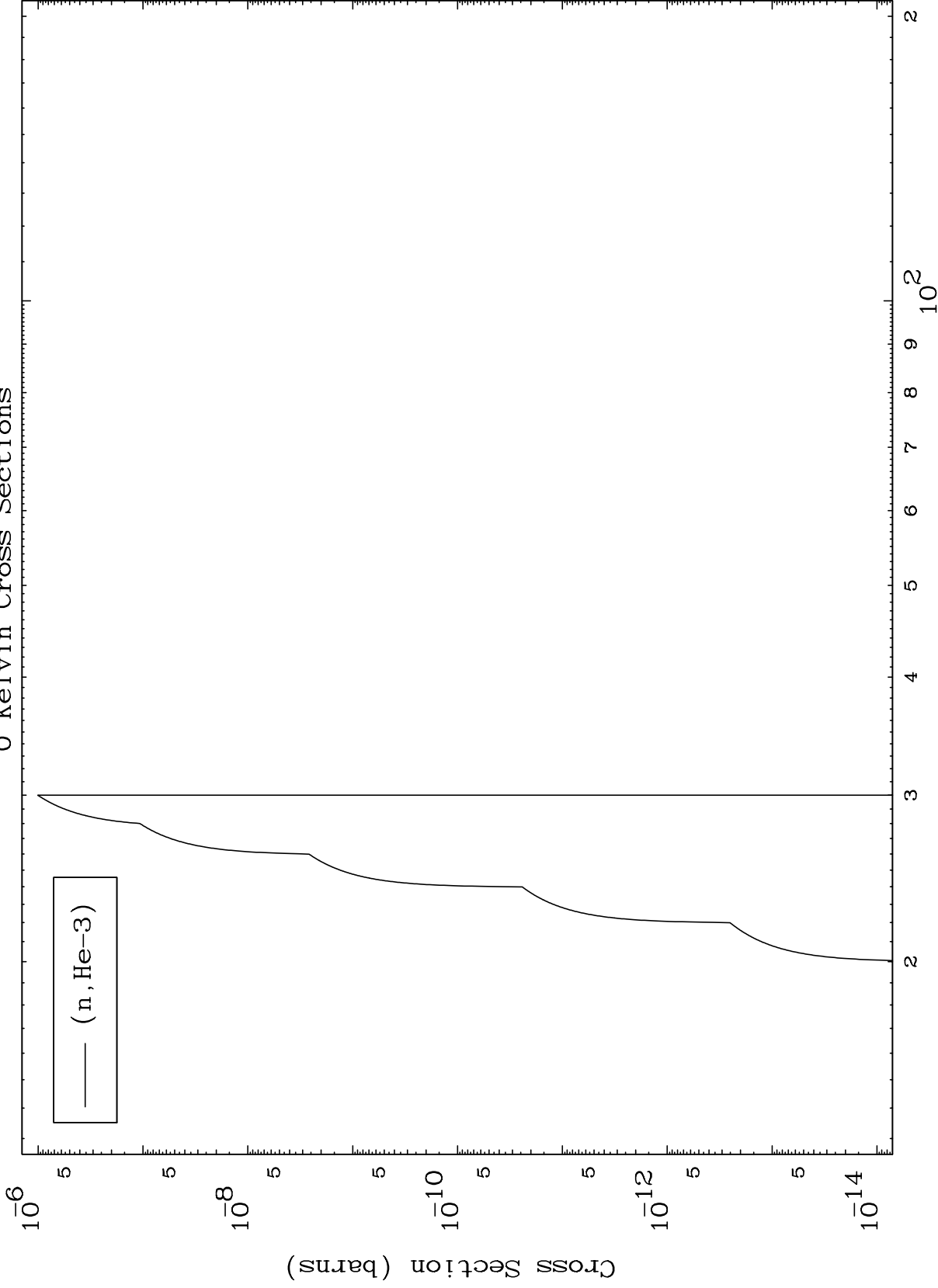
Incident Energy (MeV)

70-Yb-158

MAT 6995

70-Yb-158

( $\alpha, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

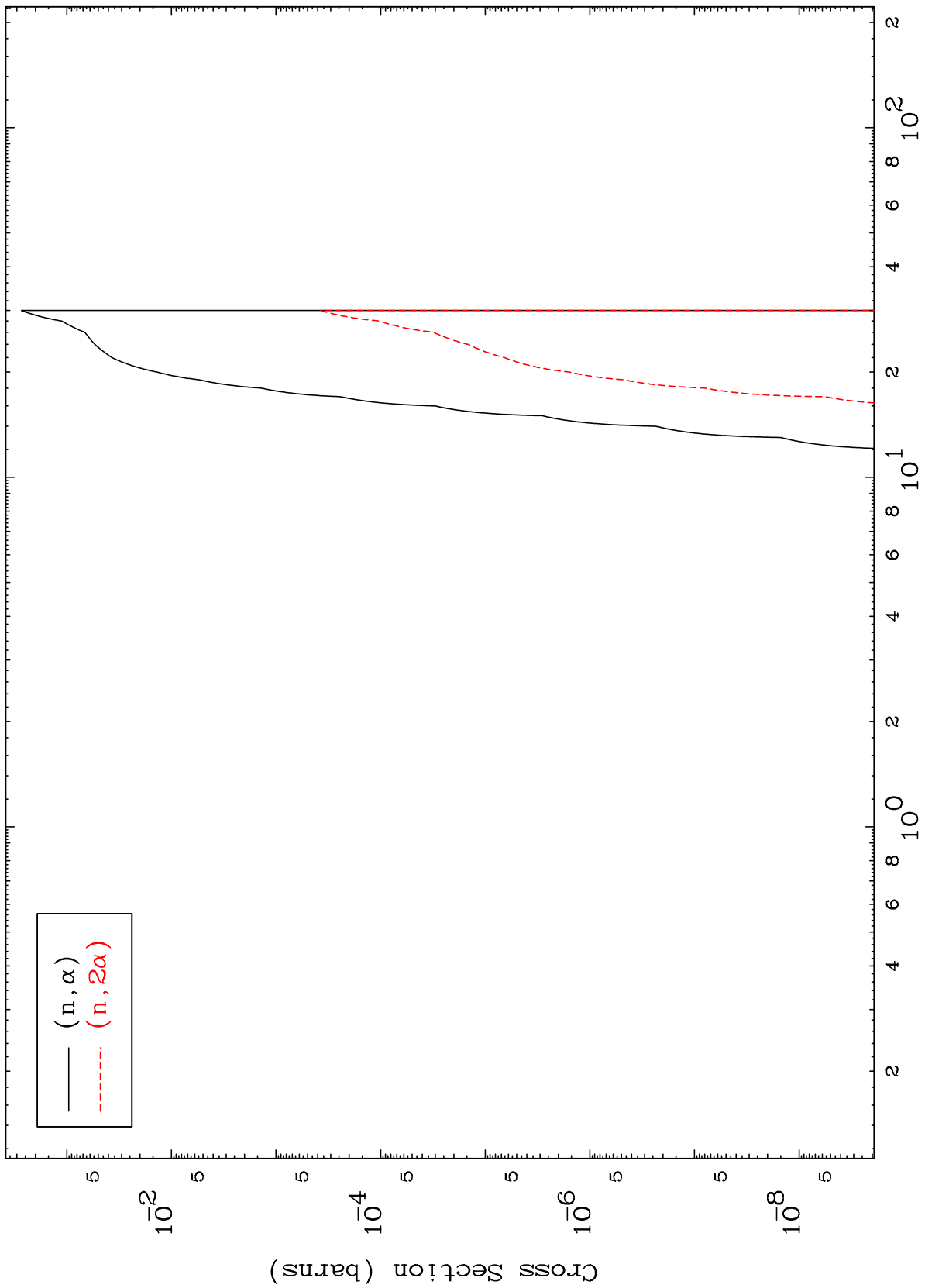


MAT 6995

( $\alpha, \alpha$ ) Levels

70-Yb-158

0 Kelvin Cross Sections



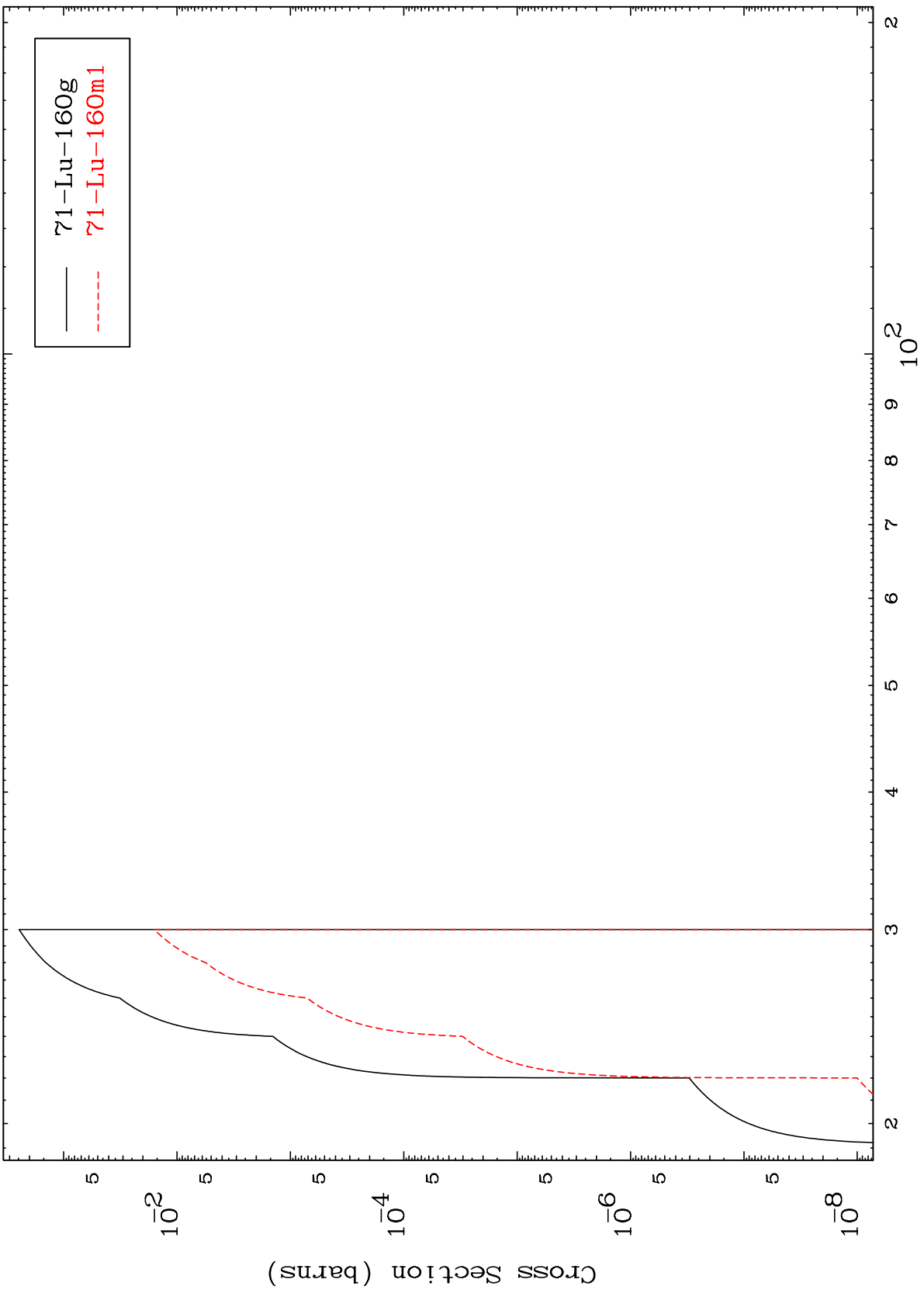
— ( $n, \alpha$ )  
- - - ( $n, 2\alpha$ )

10

Incident Energy (MeV)

70-Yb-158

Radionuclide Production Cross Section



Radionuclide Production Cross Section

