

Program EVALPLOT  
(Version 2021-1)

by

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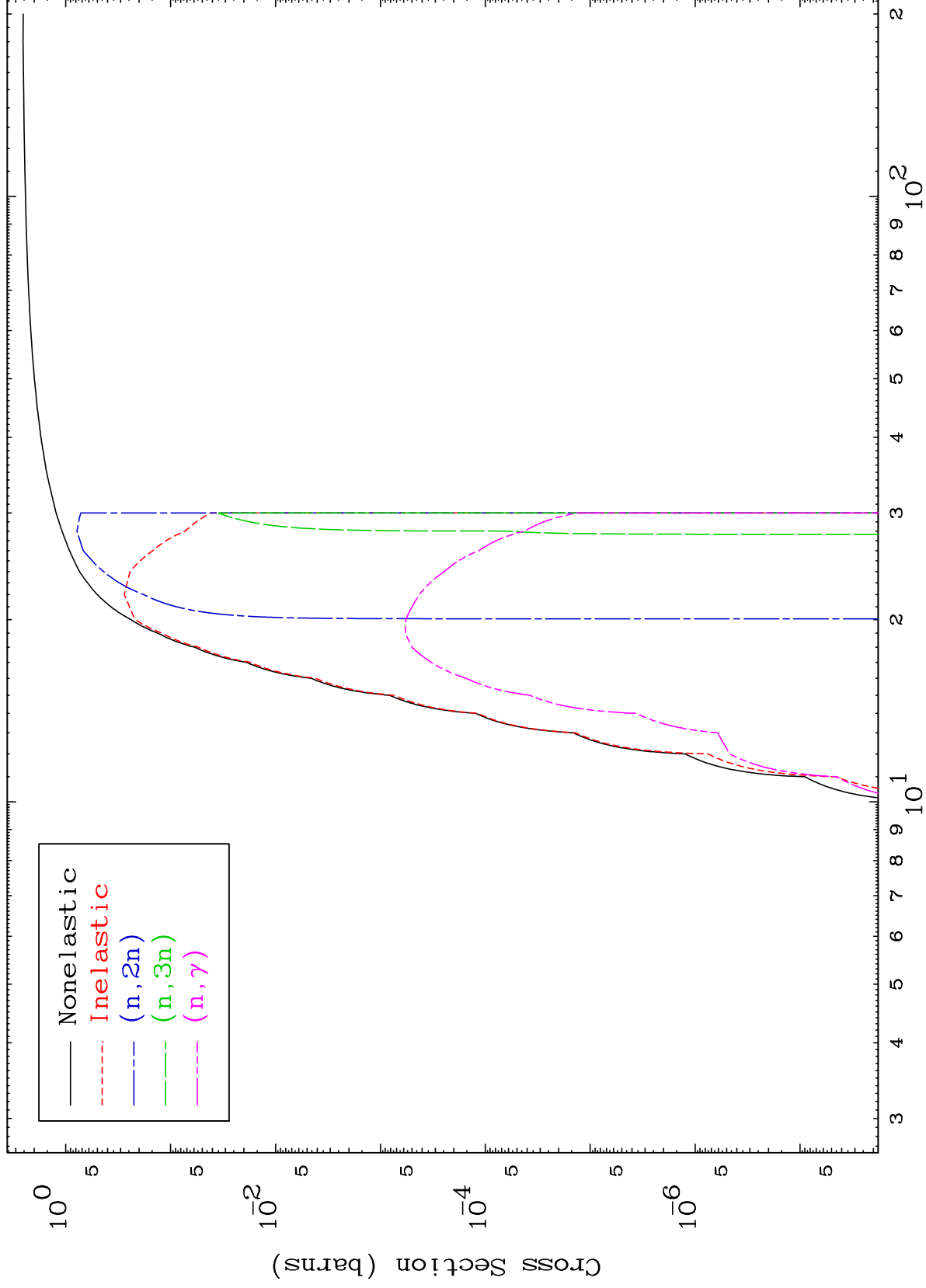
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 7022

0 Kelvin Cross Sections

70-Yb-167



1

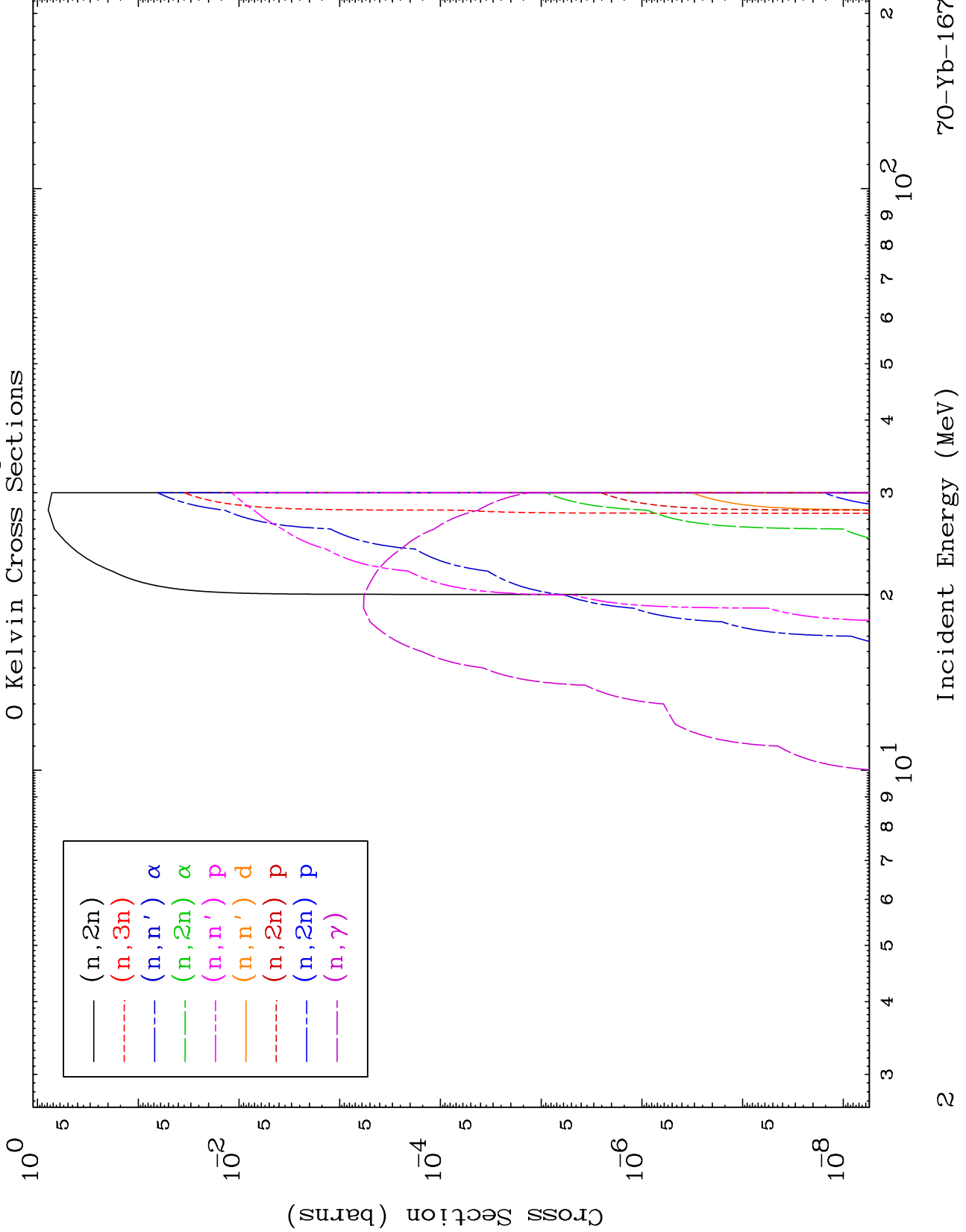
Incident Energy (MeV)

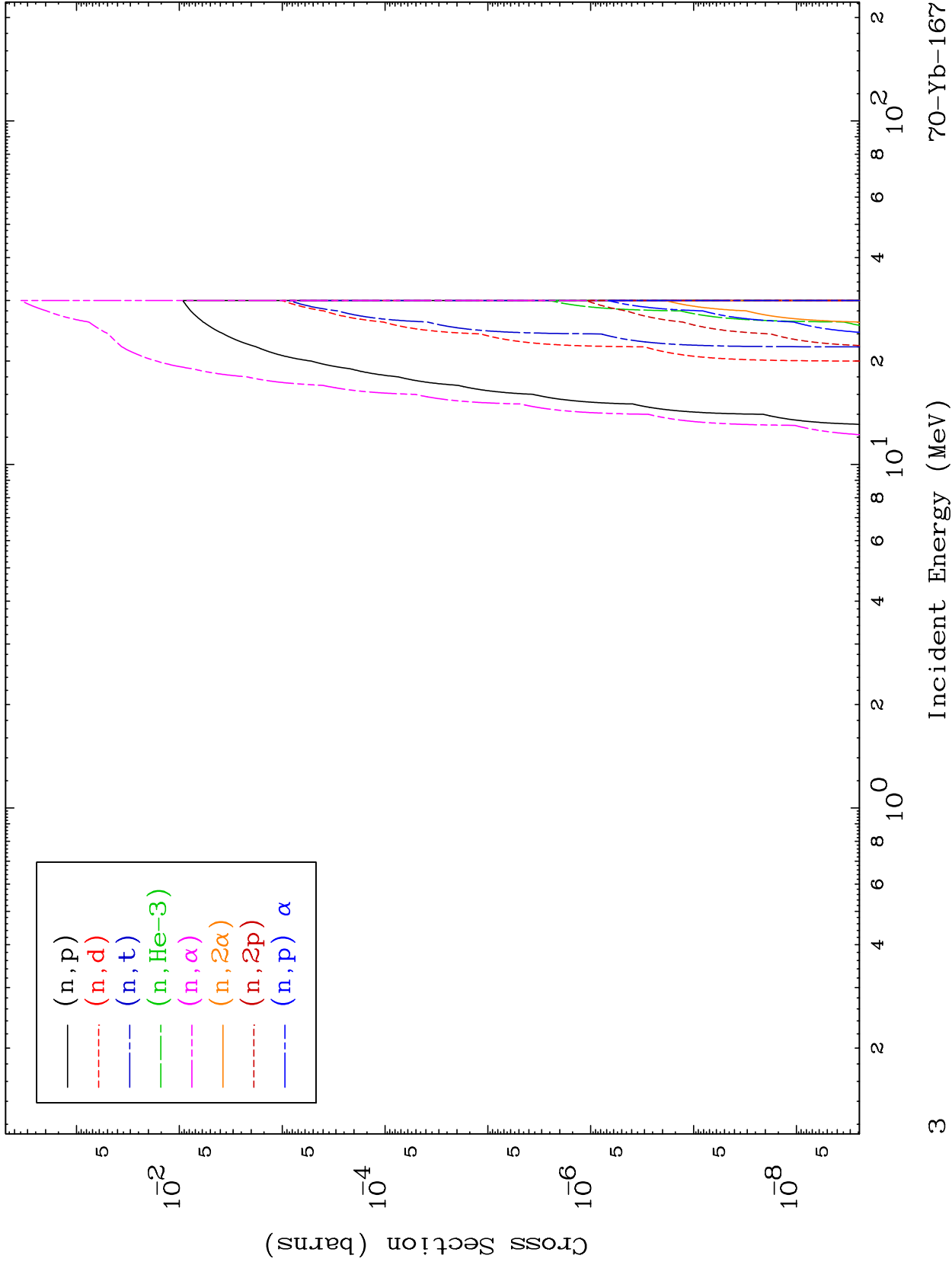
70-Yb-167

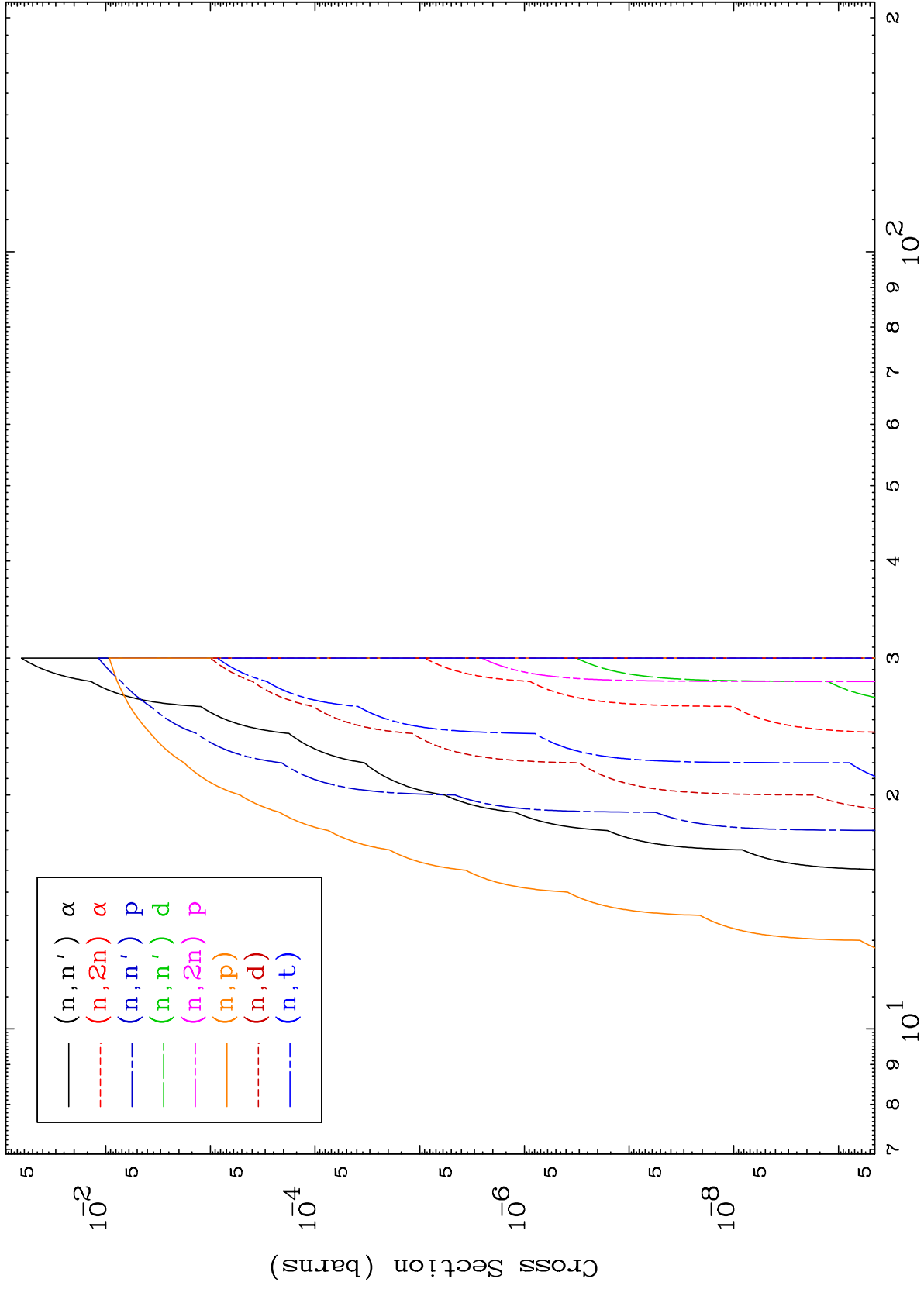
MAT 7022

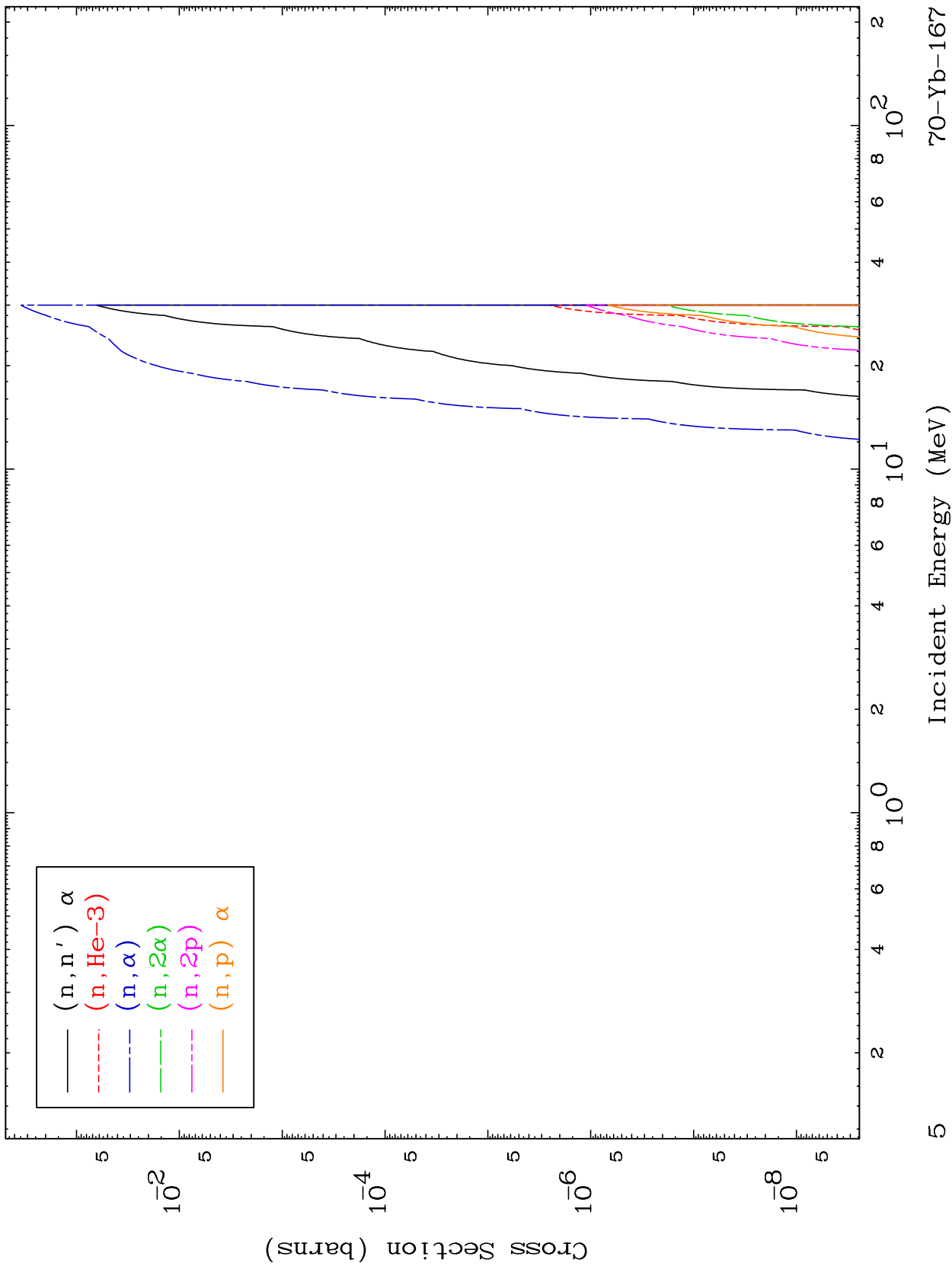
$\alpha$  Neutron Absorption  
0 Kelvin Cross Sections

70-Yb-167





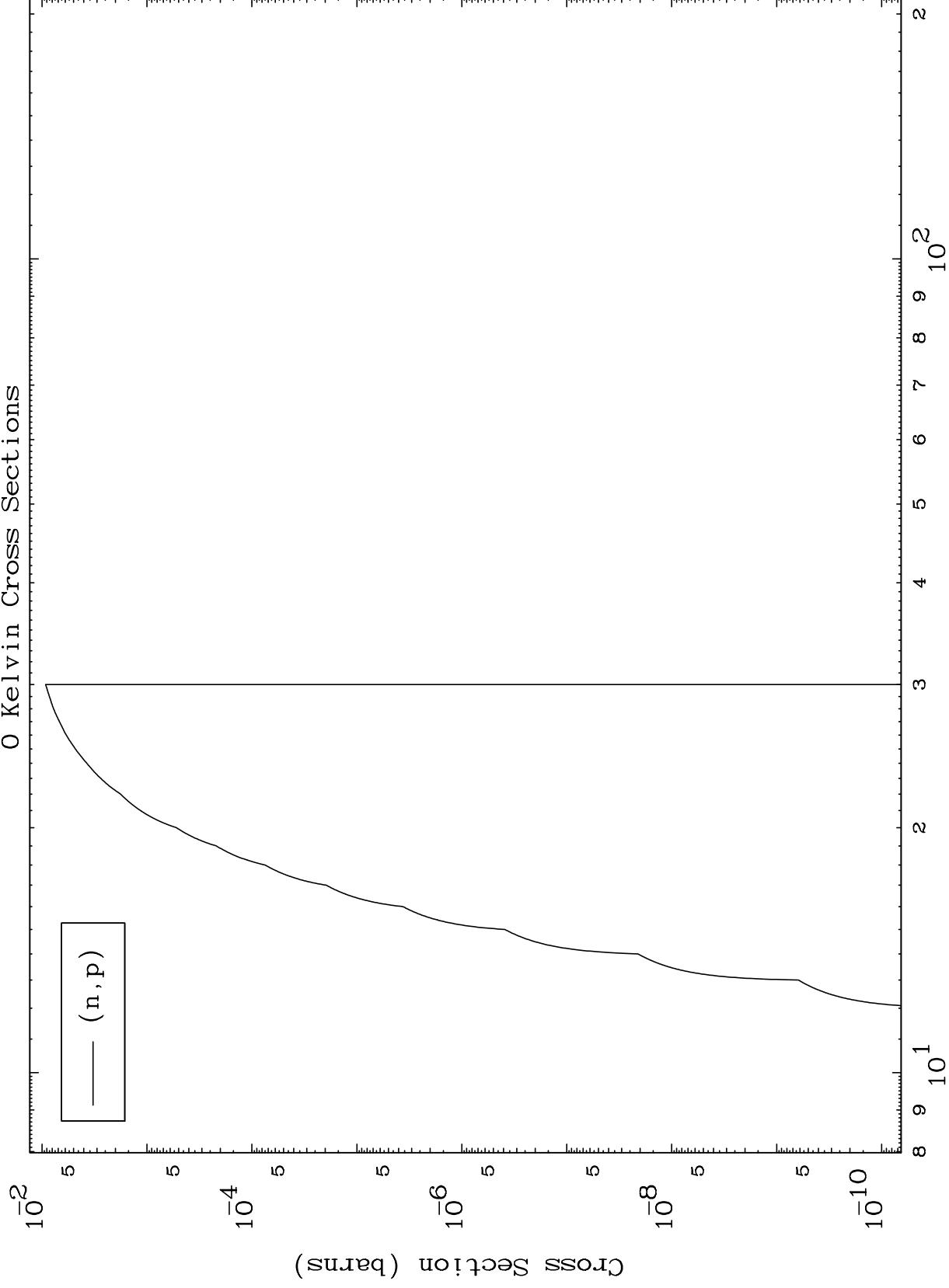




MAT 7022

( $\alpha, p$ ) Levels  
0 Kelvin Cross Sections

70-Yb-167



Incident Energy (MeV)

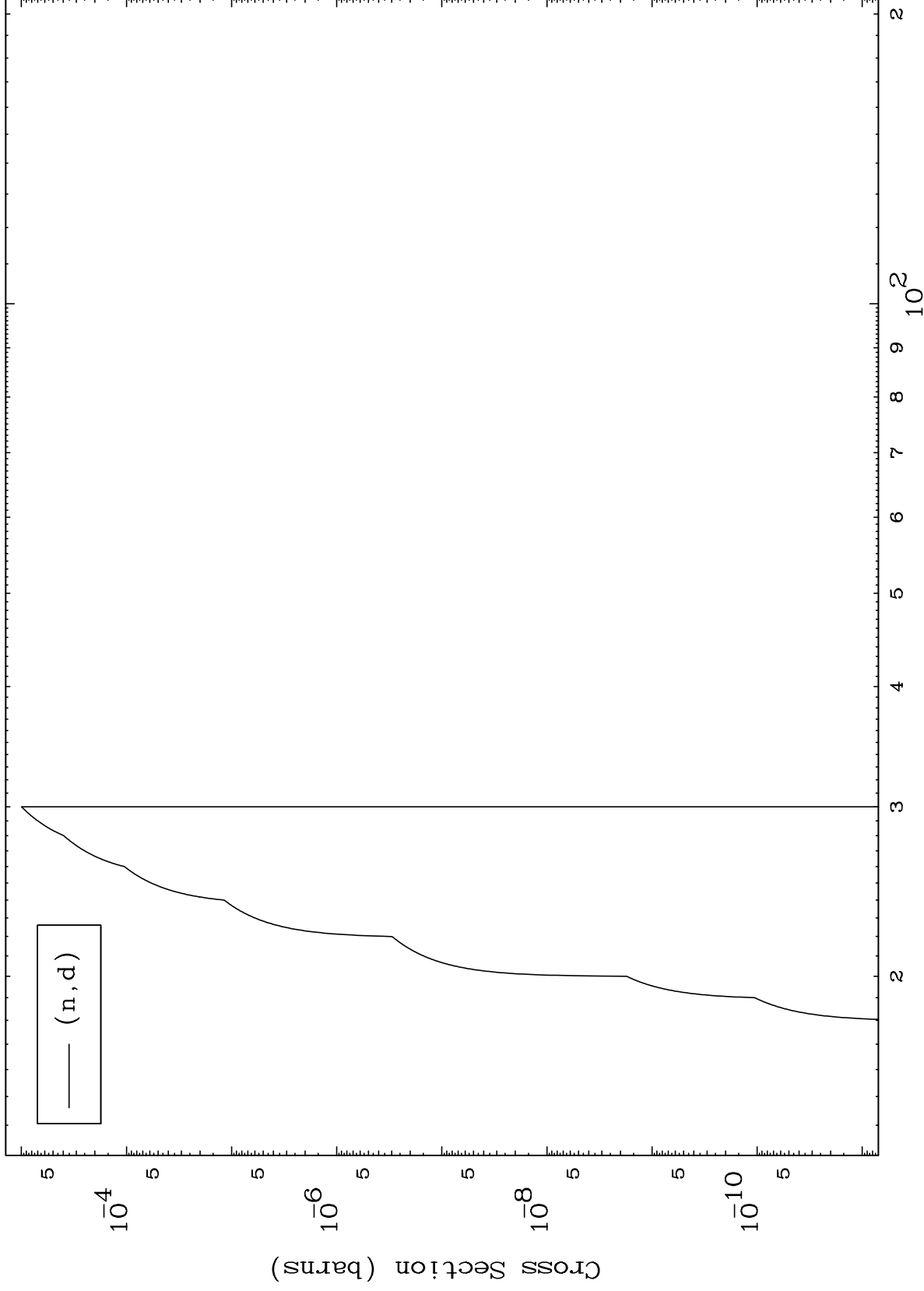
70-Yb-167

6

MAT 7022

( $\alpha, d$ ) Levels  
0 Kelvin Cross Sections

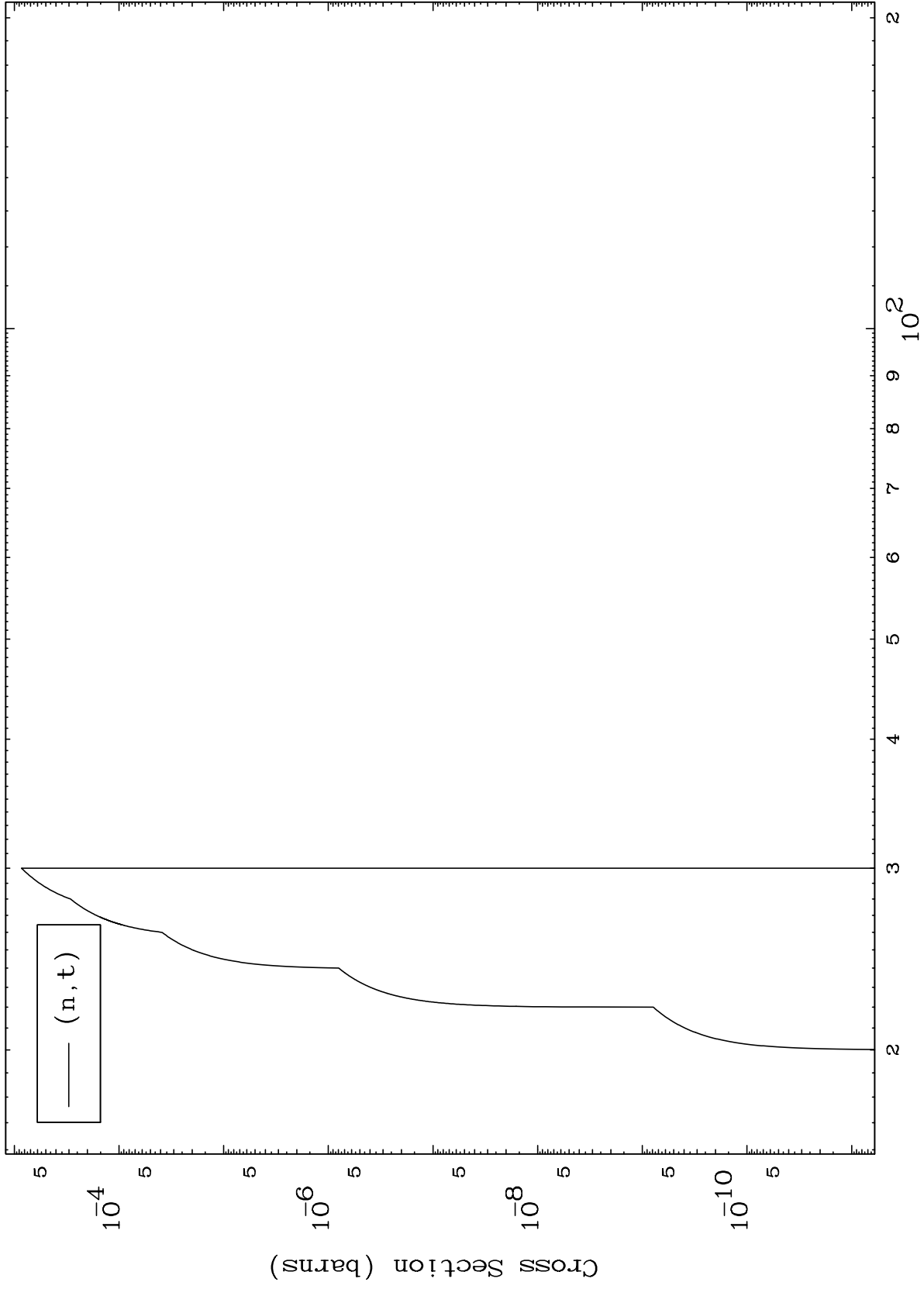
70-Yb-167

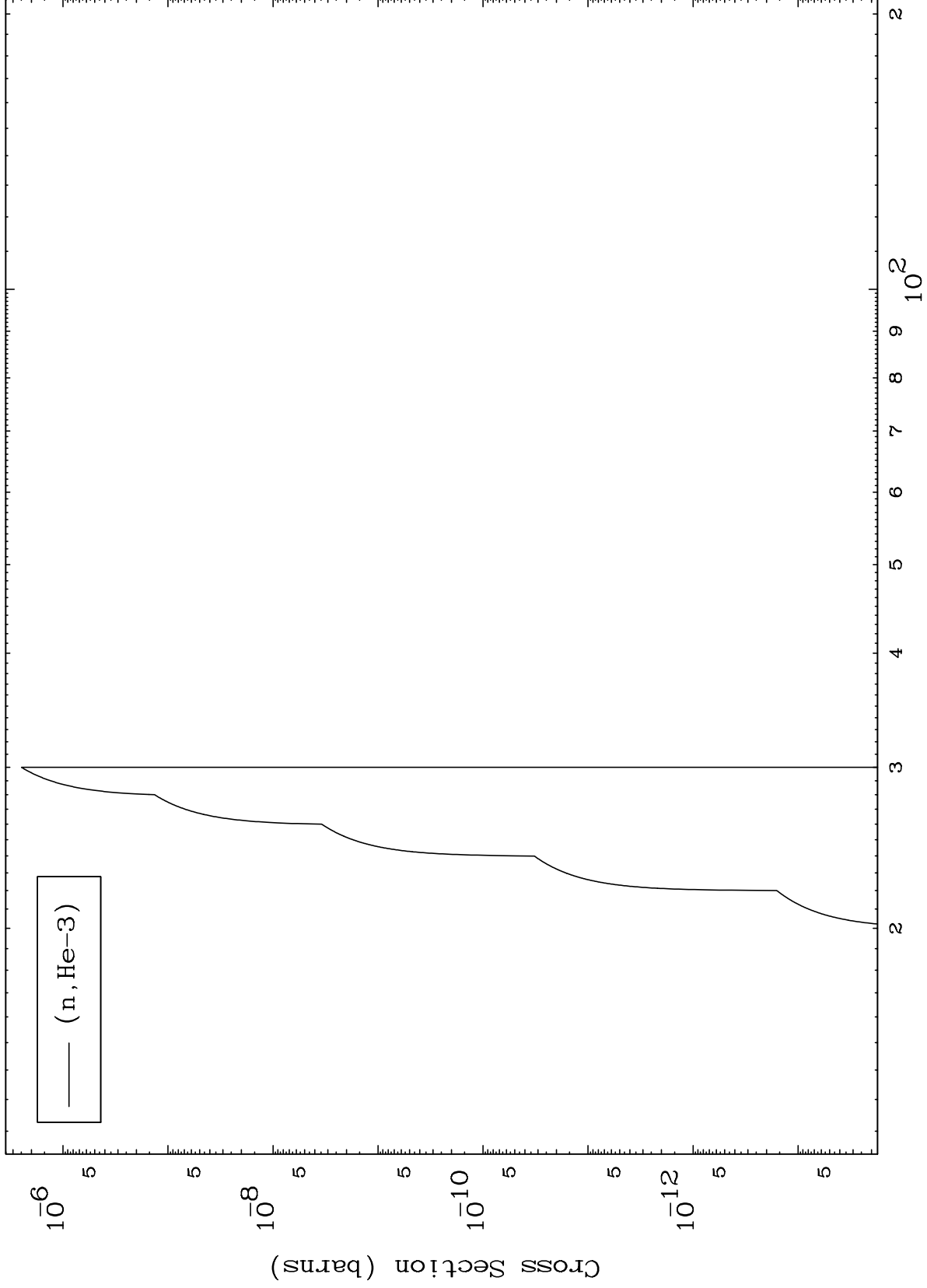


7

Incident Energy (MeV)

70-Yb-167



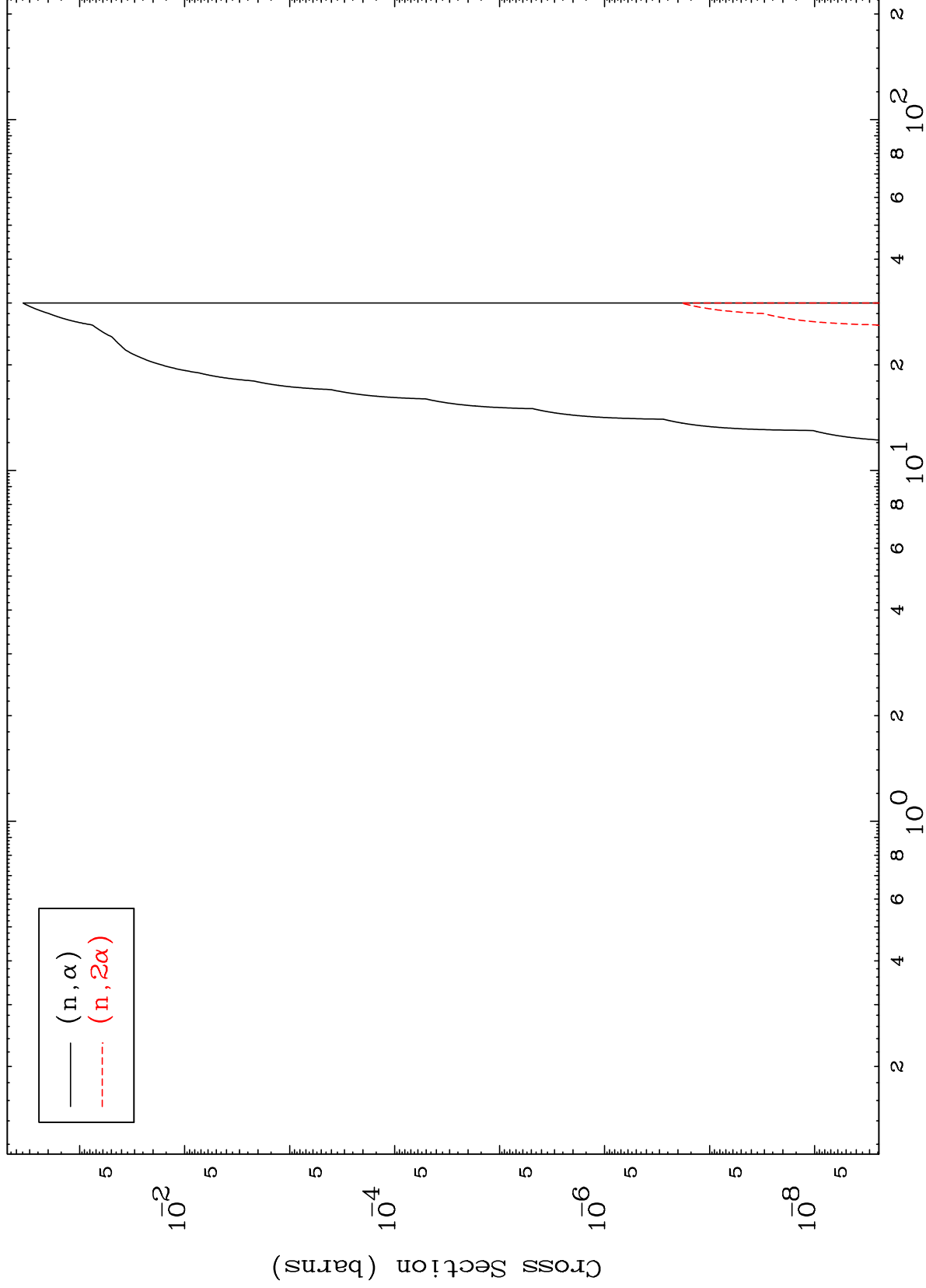


MAT 7022

( $\alpha, \alpha$ ) Levels

70-Yb-167

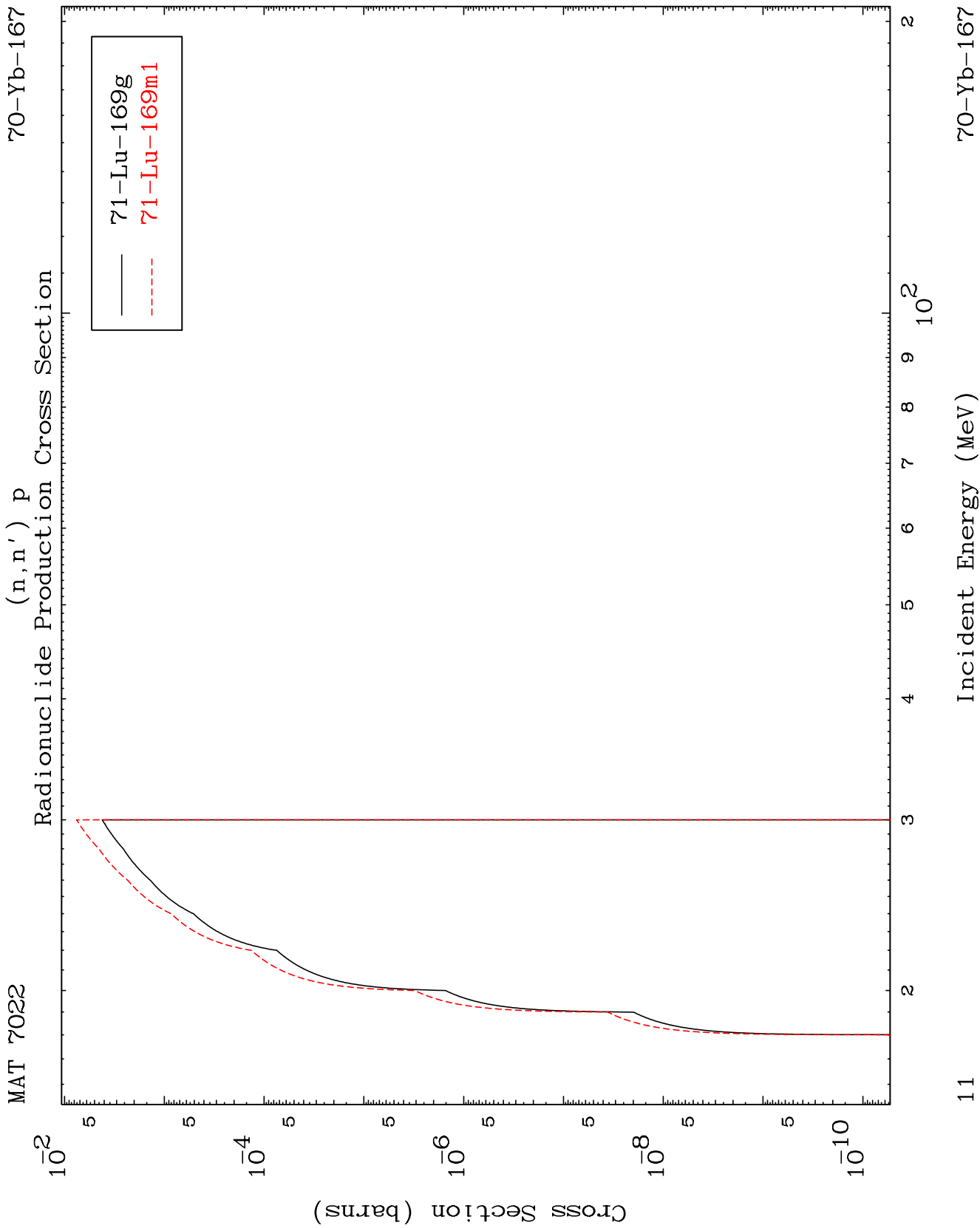
0 Kelvin Cross Sections



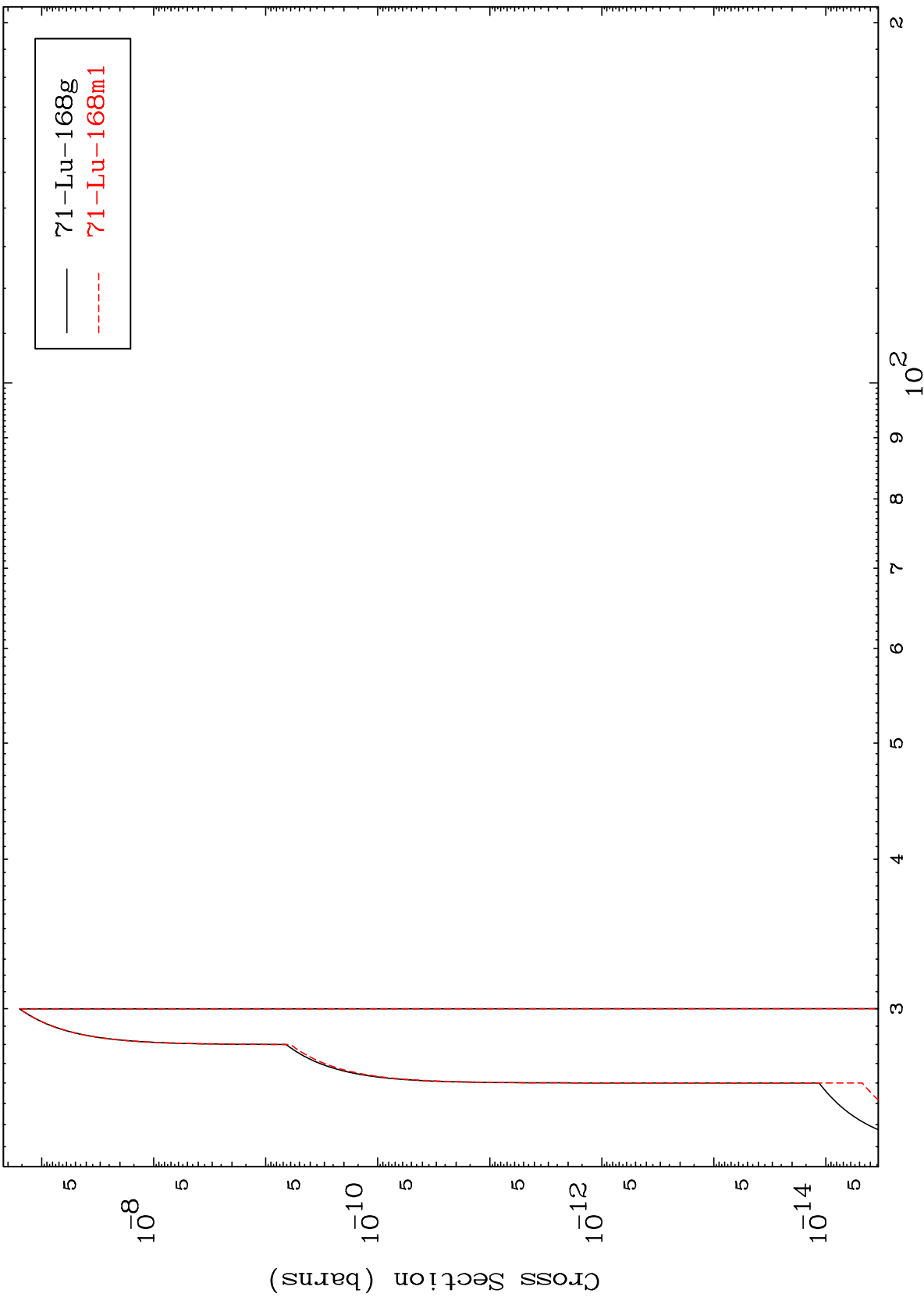
10

Incident Energy (MeV)

70-Yb-167



Radionuclide Production Cross Section



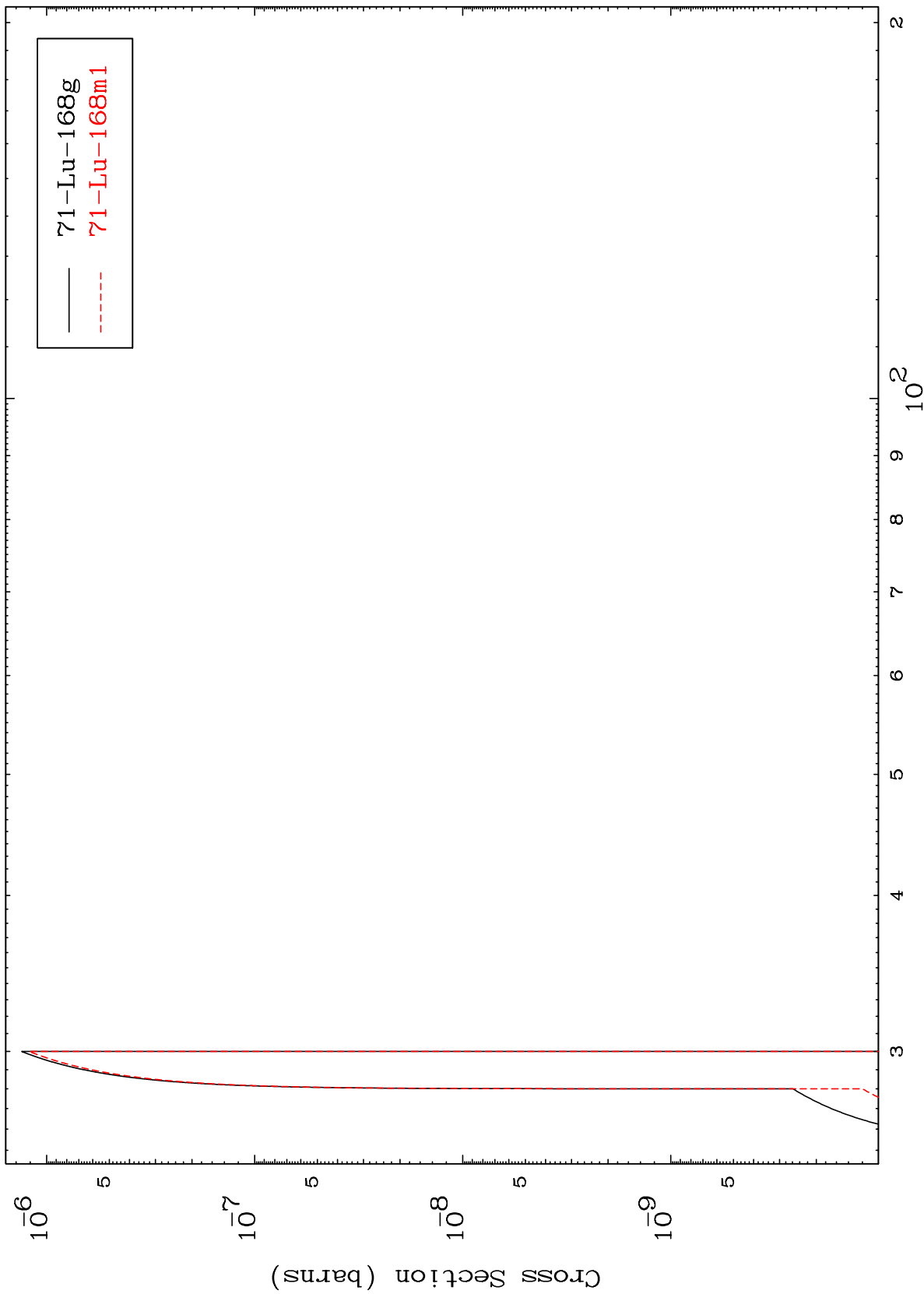
71-Lu-168g  
71-Lu-168m1

MAT 7022

70-Yb-167

(n,2n) p

Radionuclide Production Cross Section



71-Lu-168g  
71-Lu-168m1

70-Yb-167

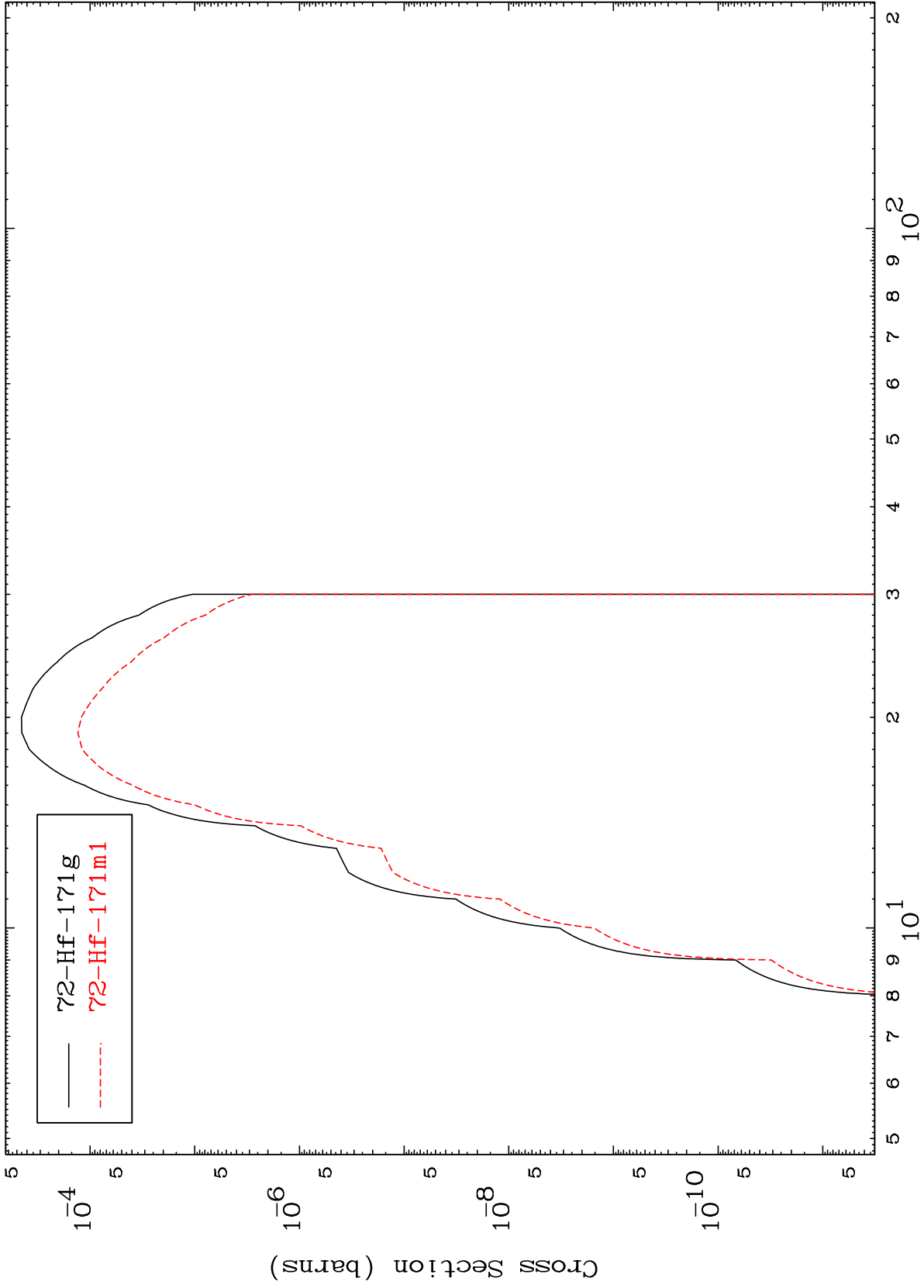
Incident Energy (MeV)

13

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70-Yb-167

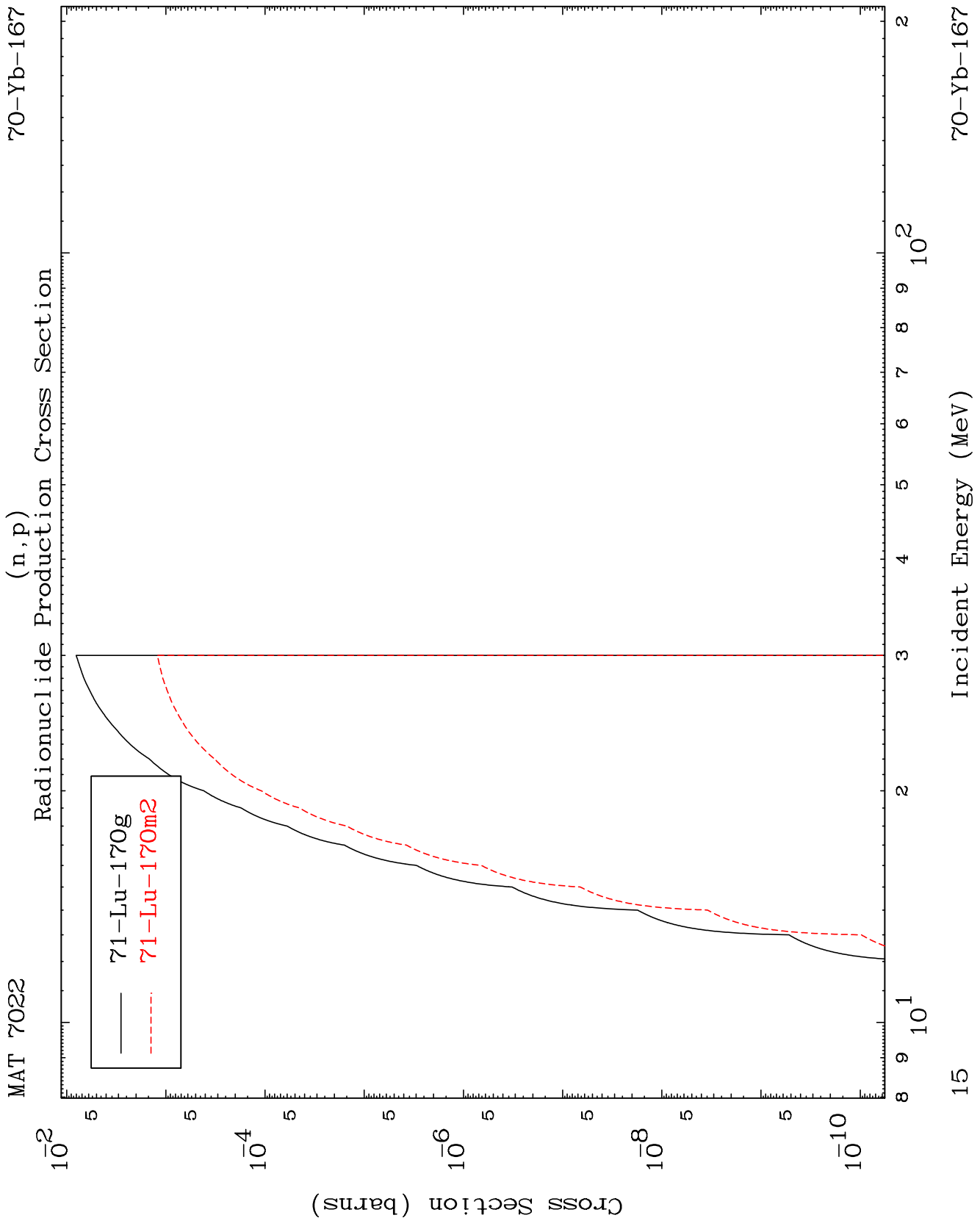
Radionuclide Production Cross Section (n,γ)



70-Yb-167

Incident Energy (MeV)

14

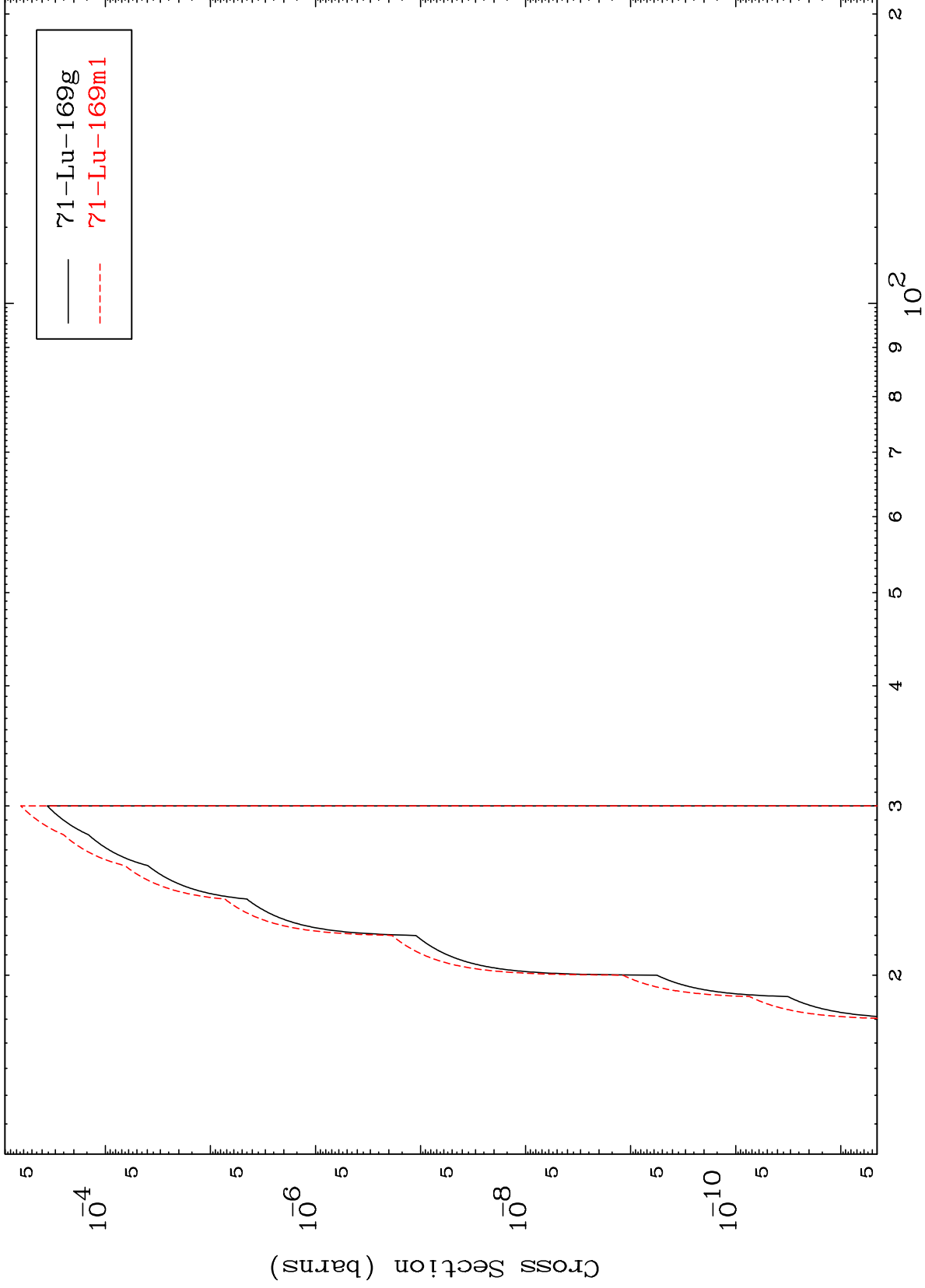


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(n,d)

70-Yb-167

Radionuclide Production Cross Section

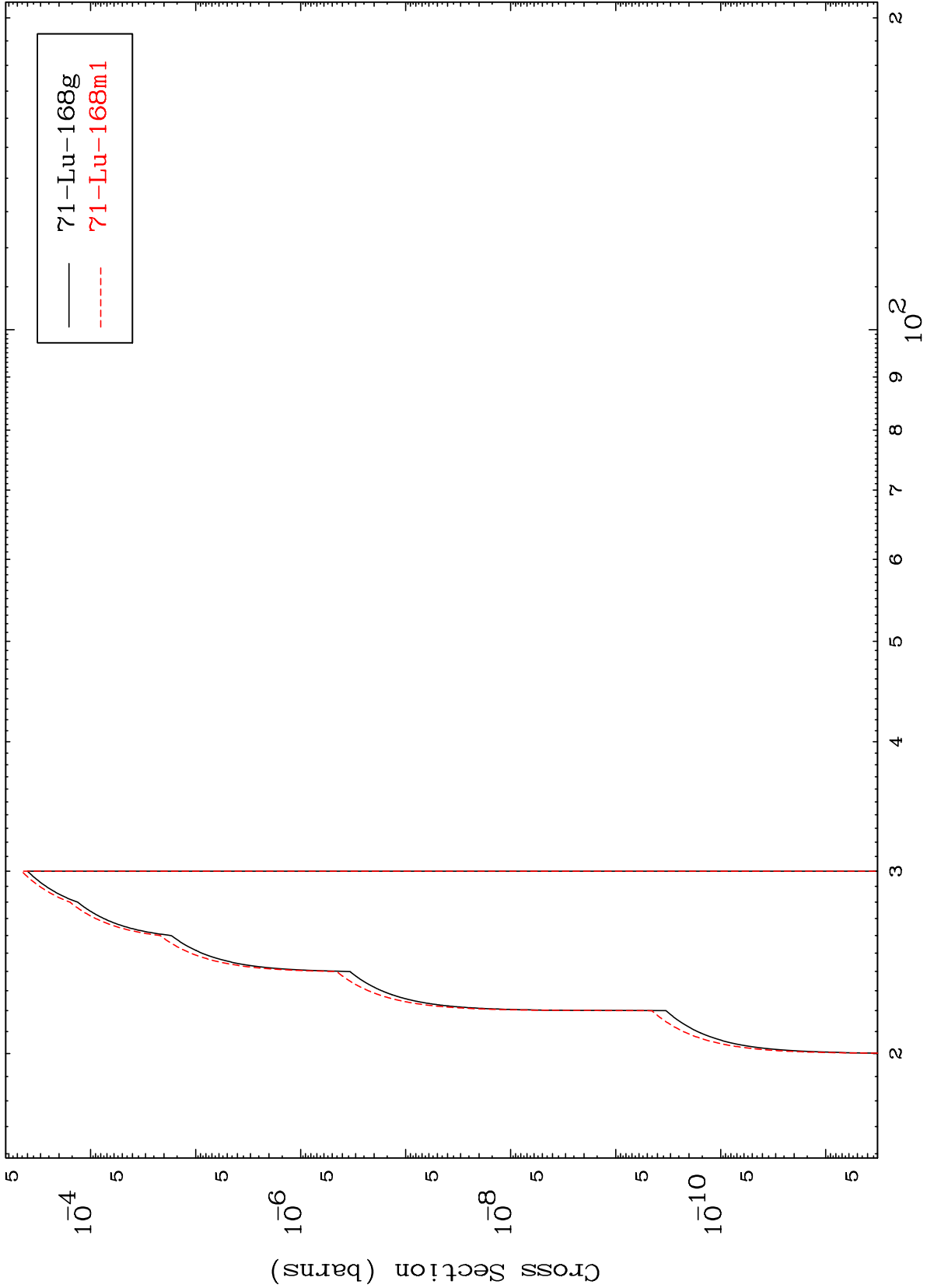


16

Incident Energy (MeV)

70-Yb-167

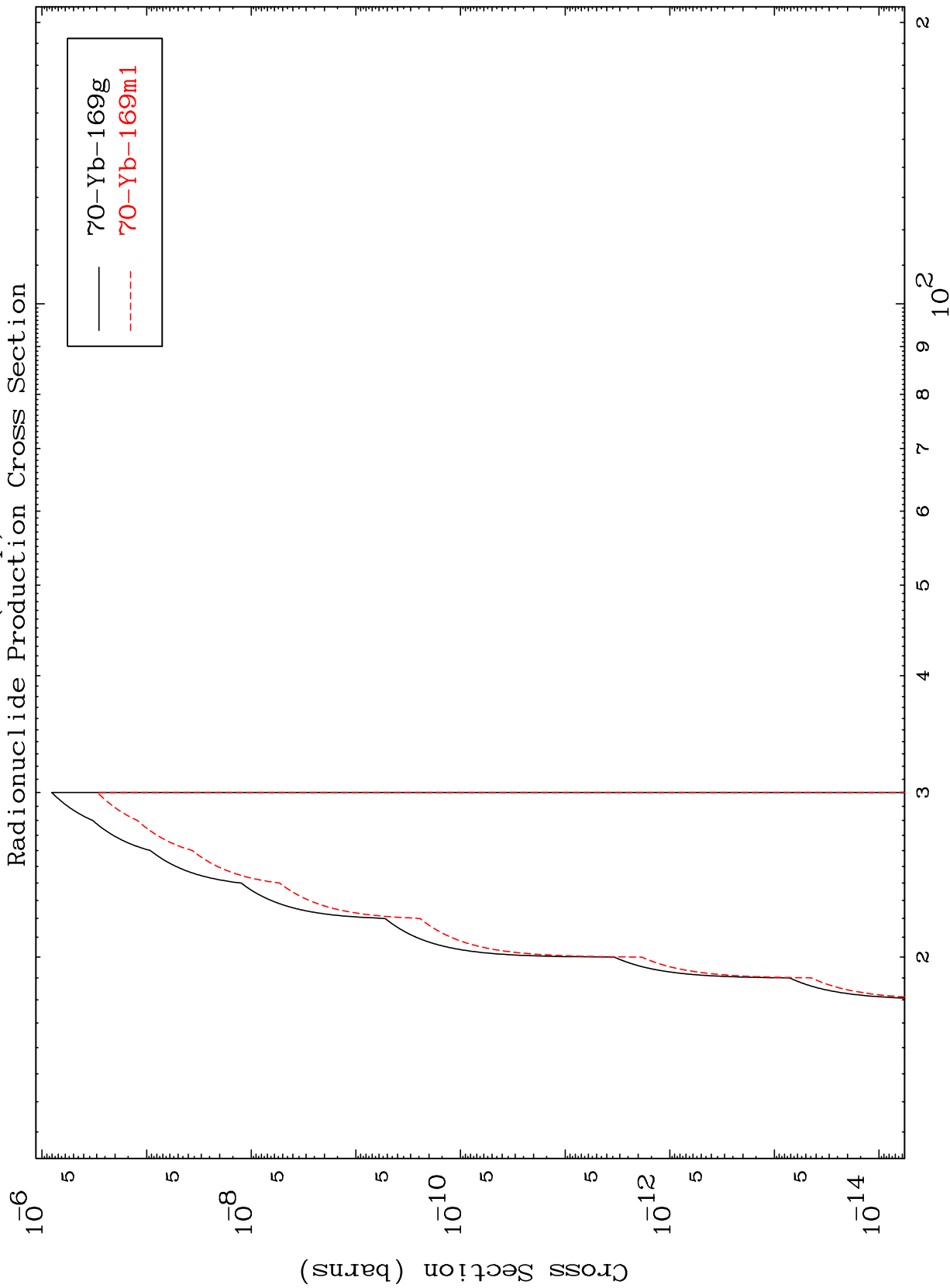
(n, t)  
Radionuclide Production Cross Section



MAT 7022

70-Yb-167

(n,2p)  
Radionuclide Production Cross Section



18

70-Yb-167

Incident Energy (MeV)