

Program EVALPLOT  
(Version 2021-1)

by

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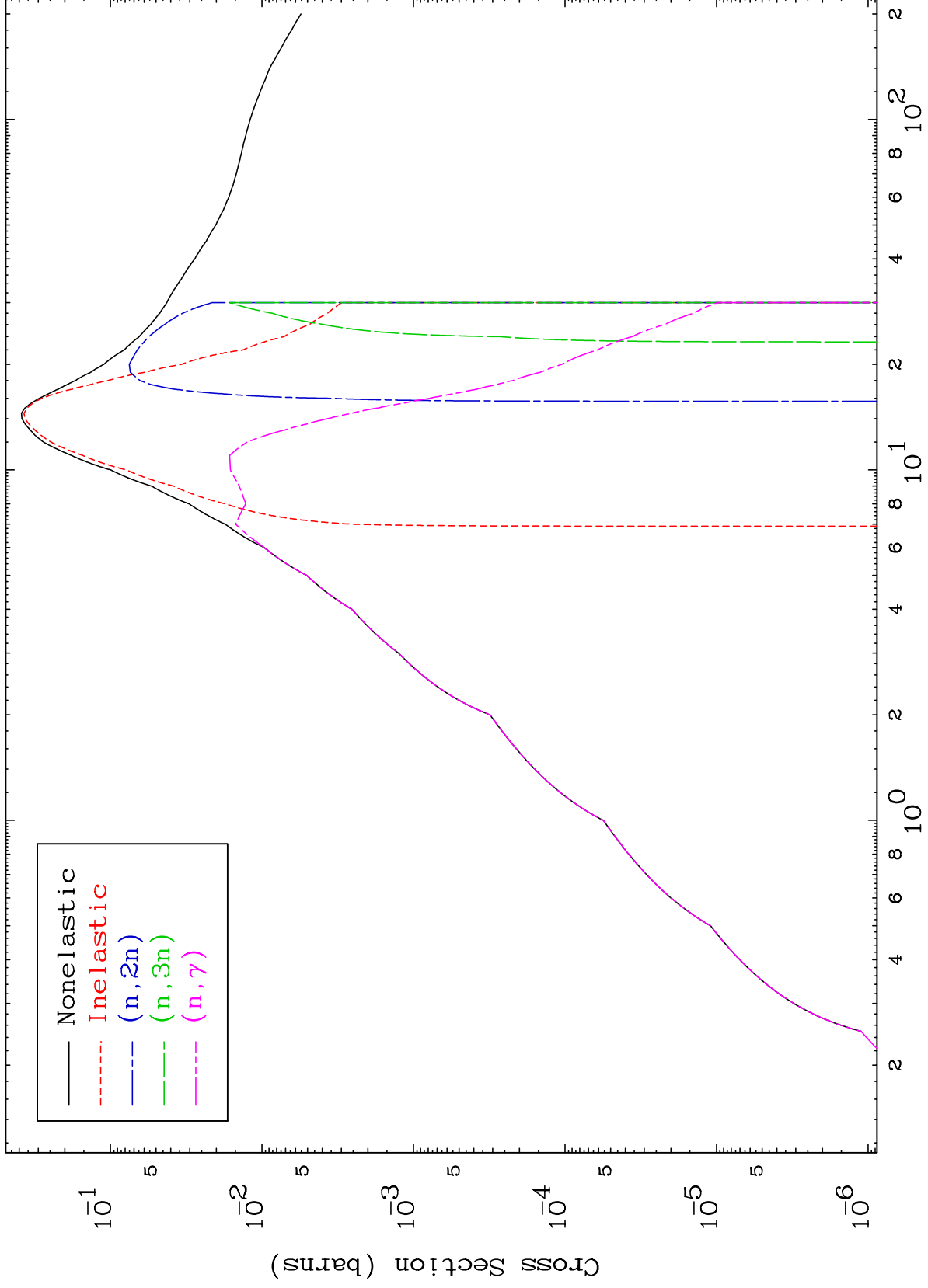
Press Mouse Button to Start

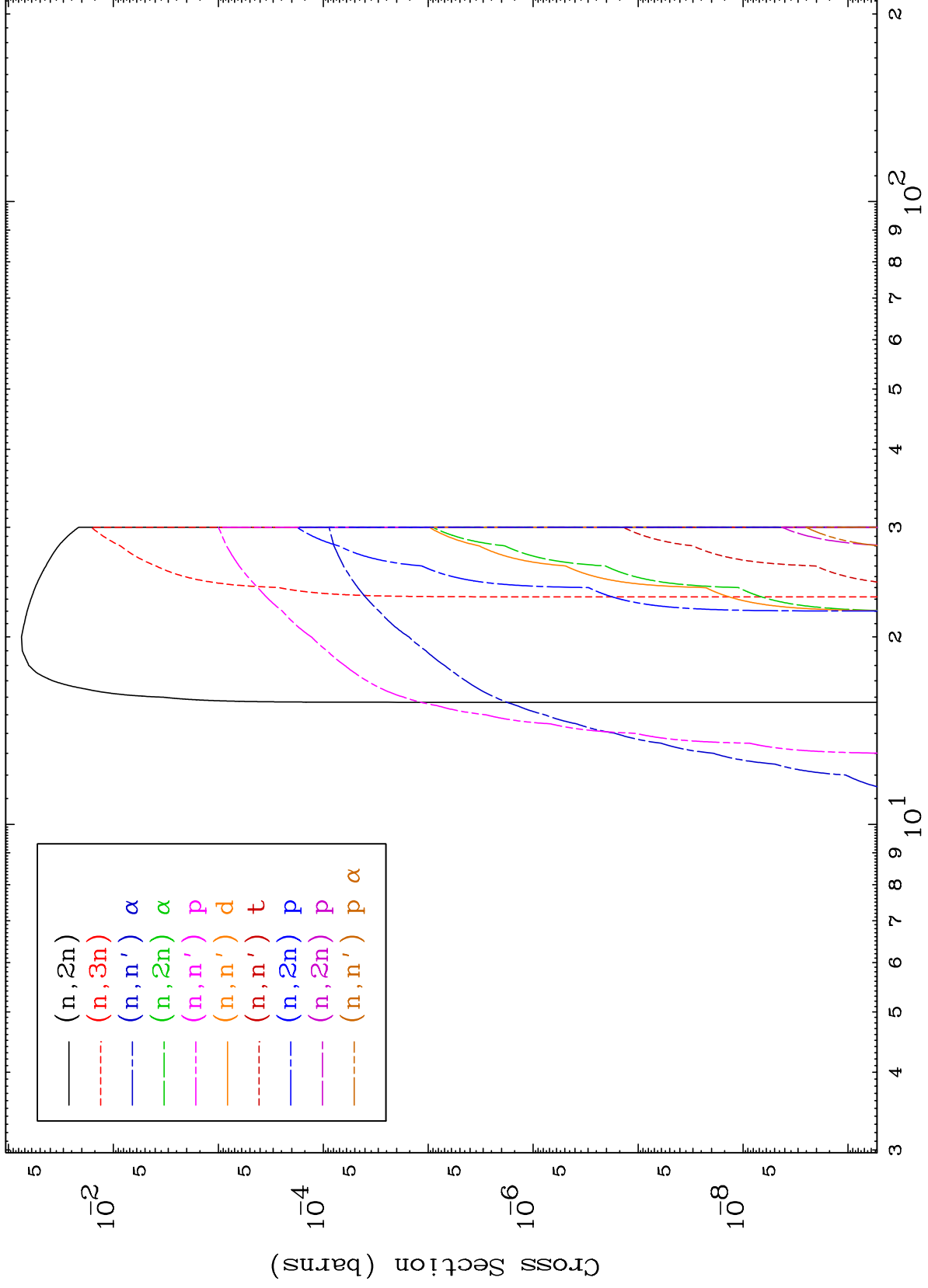
MAT 7710

Photon Major

77-Ir-186

0 Kelvin Cross Sections

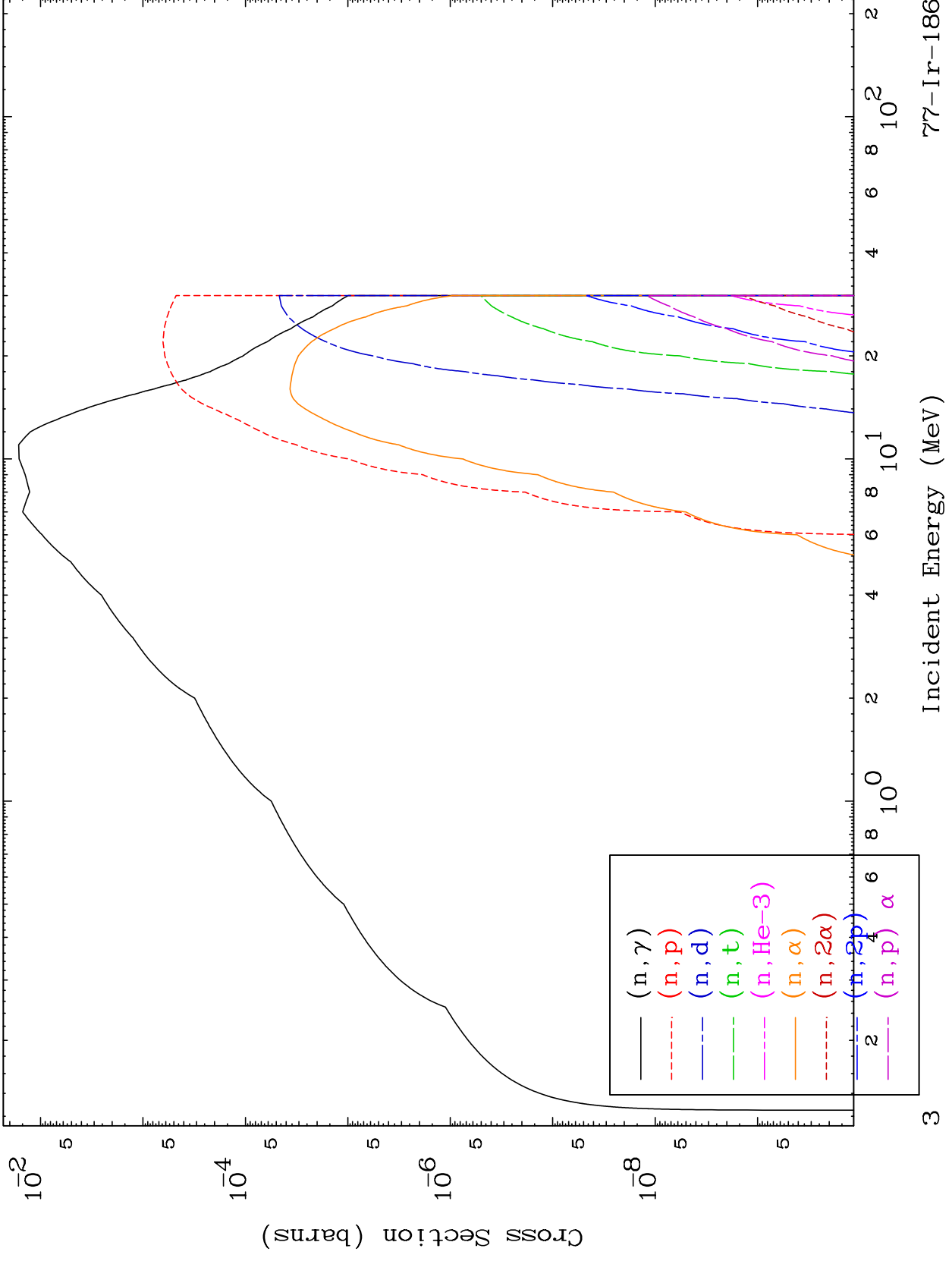


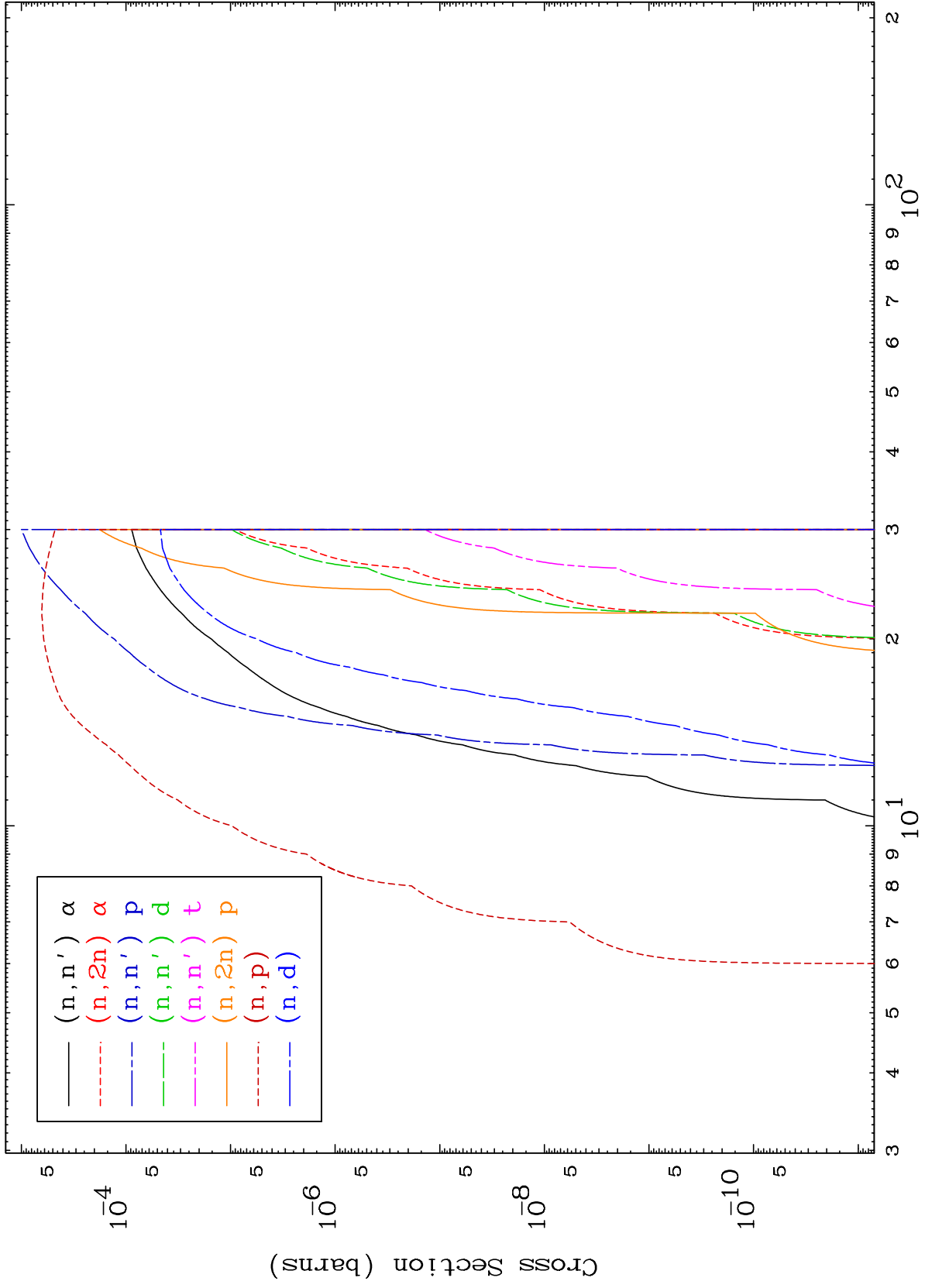


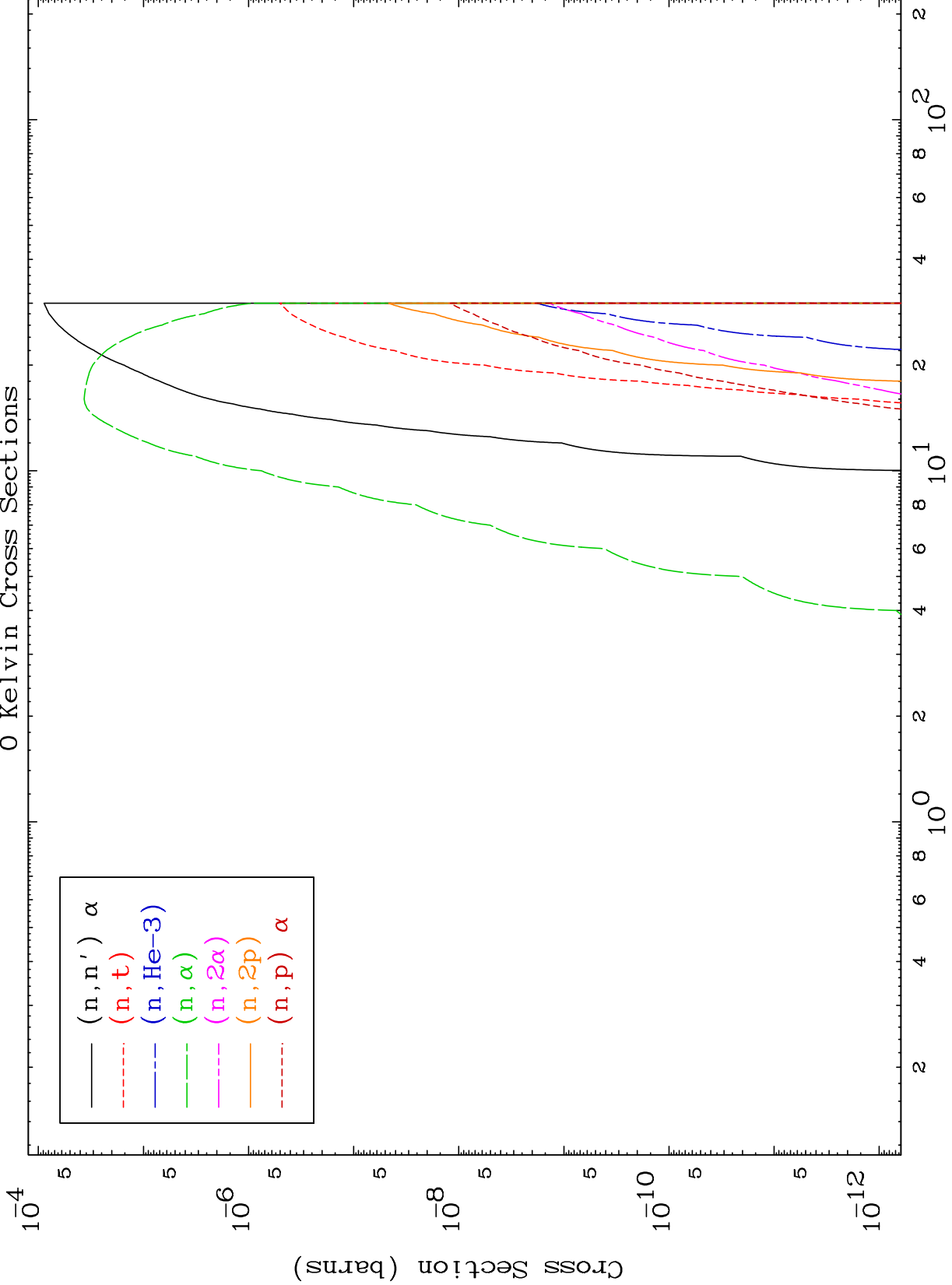
MAT 7710

Photon Neutron Absorption  
0 Kelvin Cross Sections

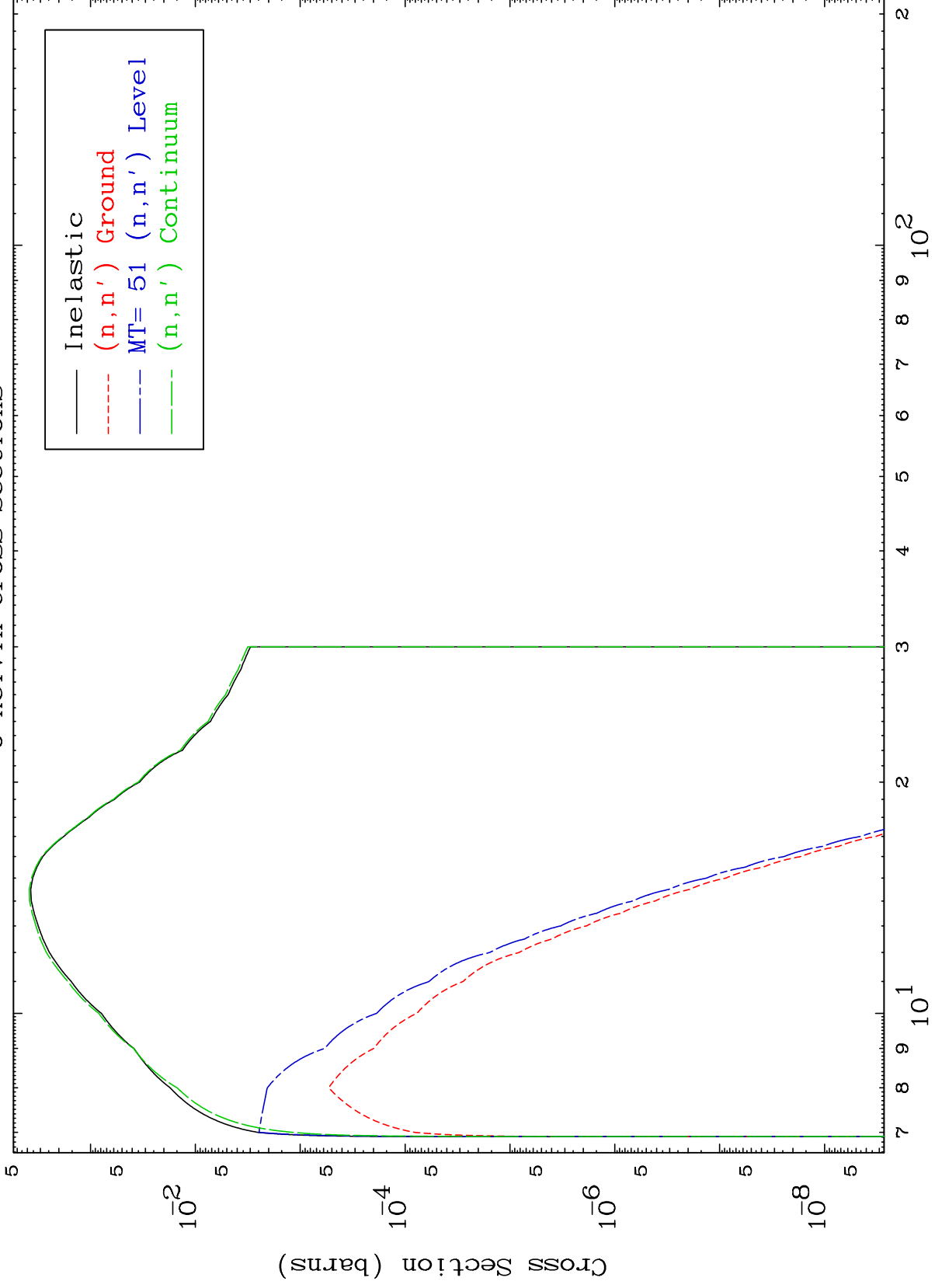
77-Ir-186







0 Kelvin Cross Sections

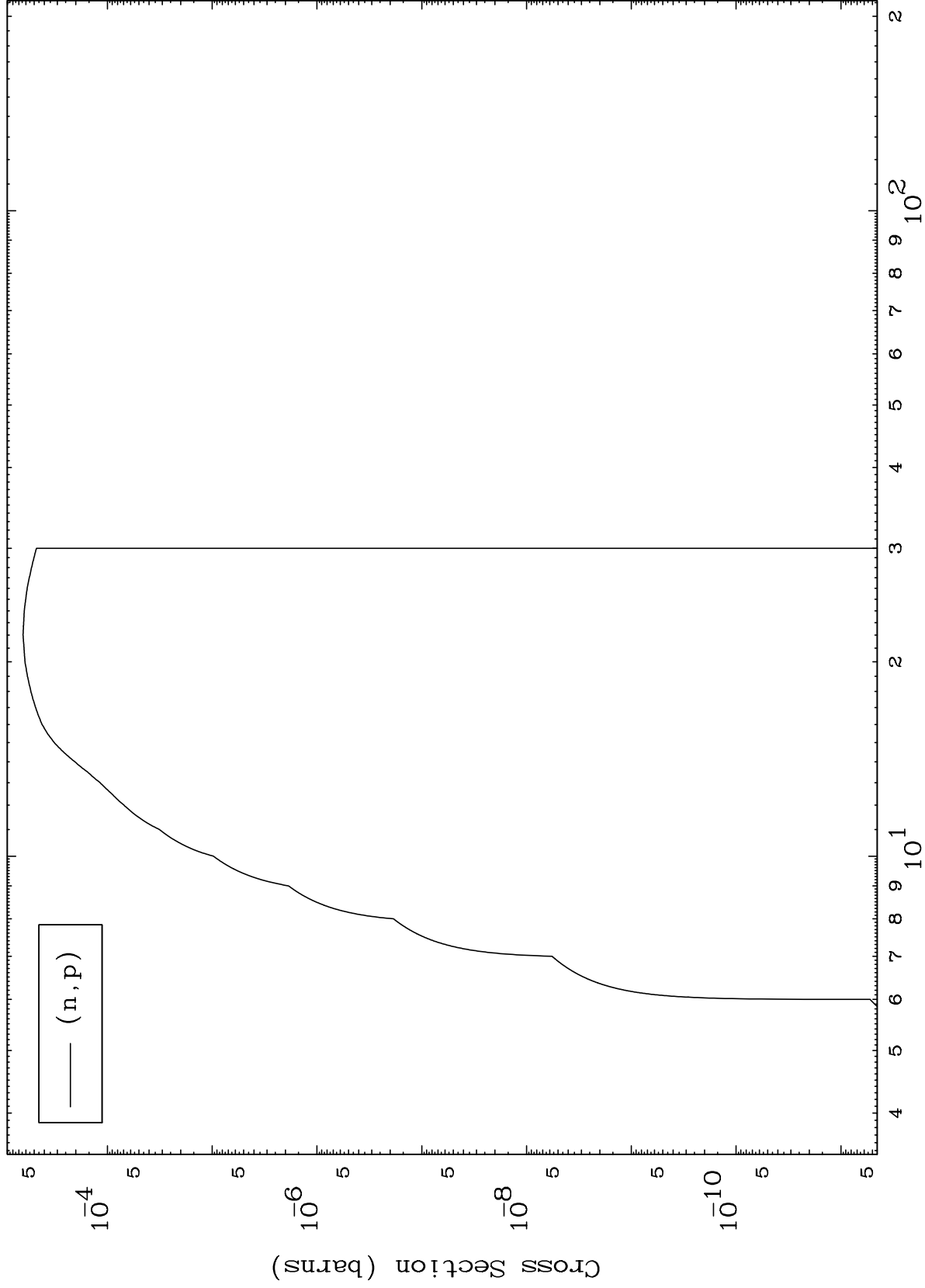


MAT 7710

( $\gamma, p$ ) Levels

77-Ir-186

0 Kelvin Cross Sections



7

Incident Energy (MeV)

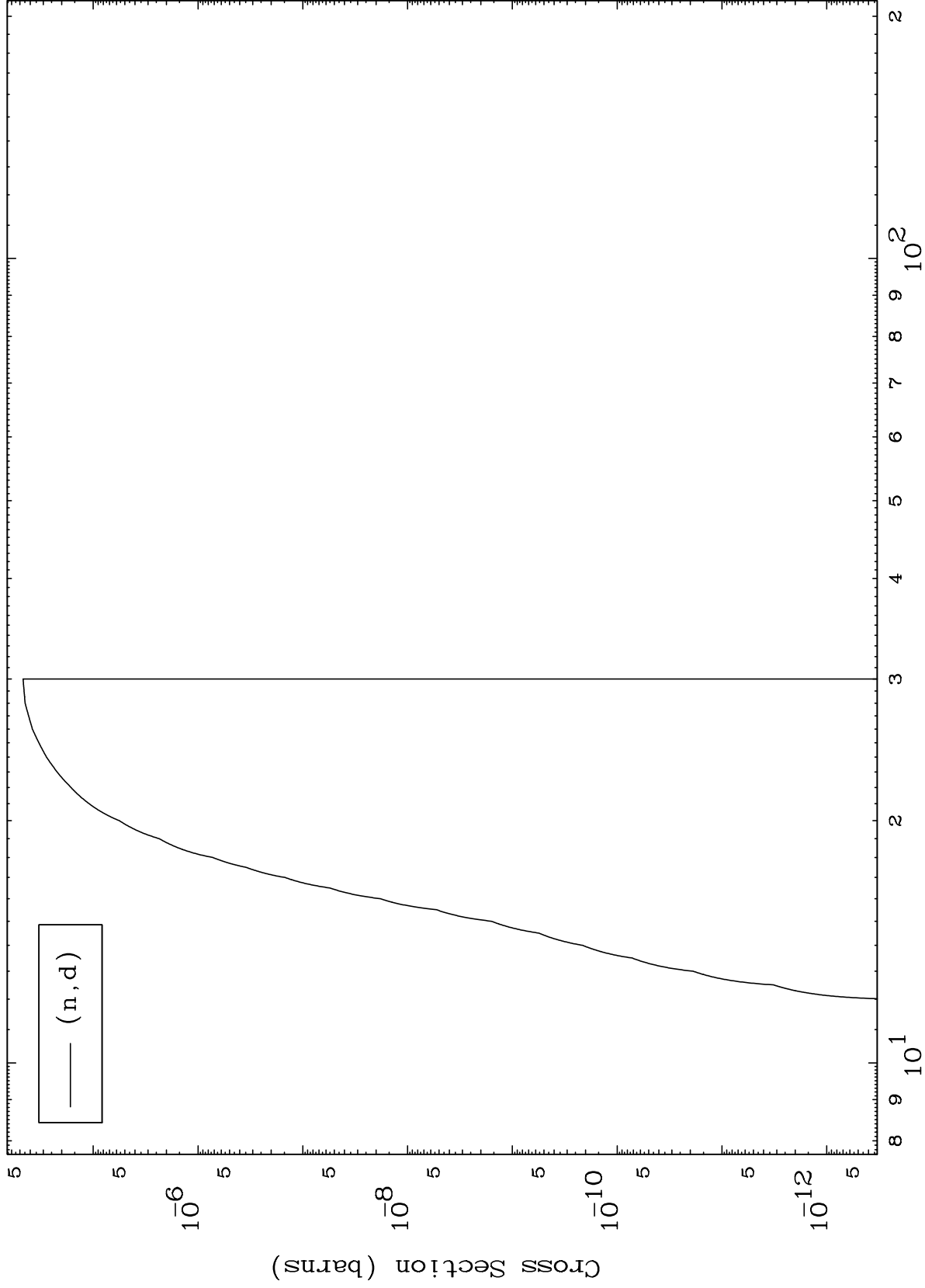
77-Ir-186

MAT 7710

( $\gamma, d$ ) Levels

77-Ir-186

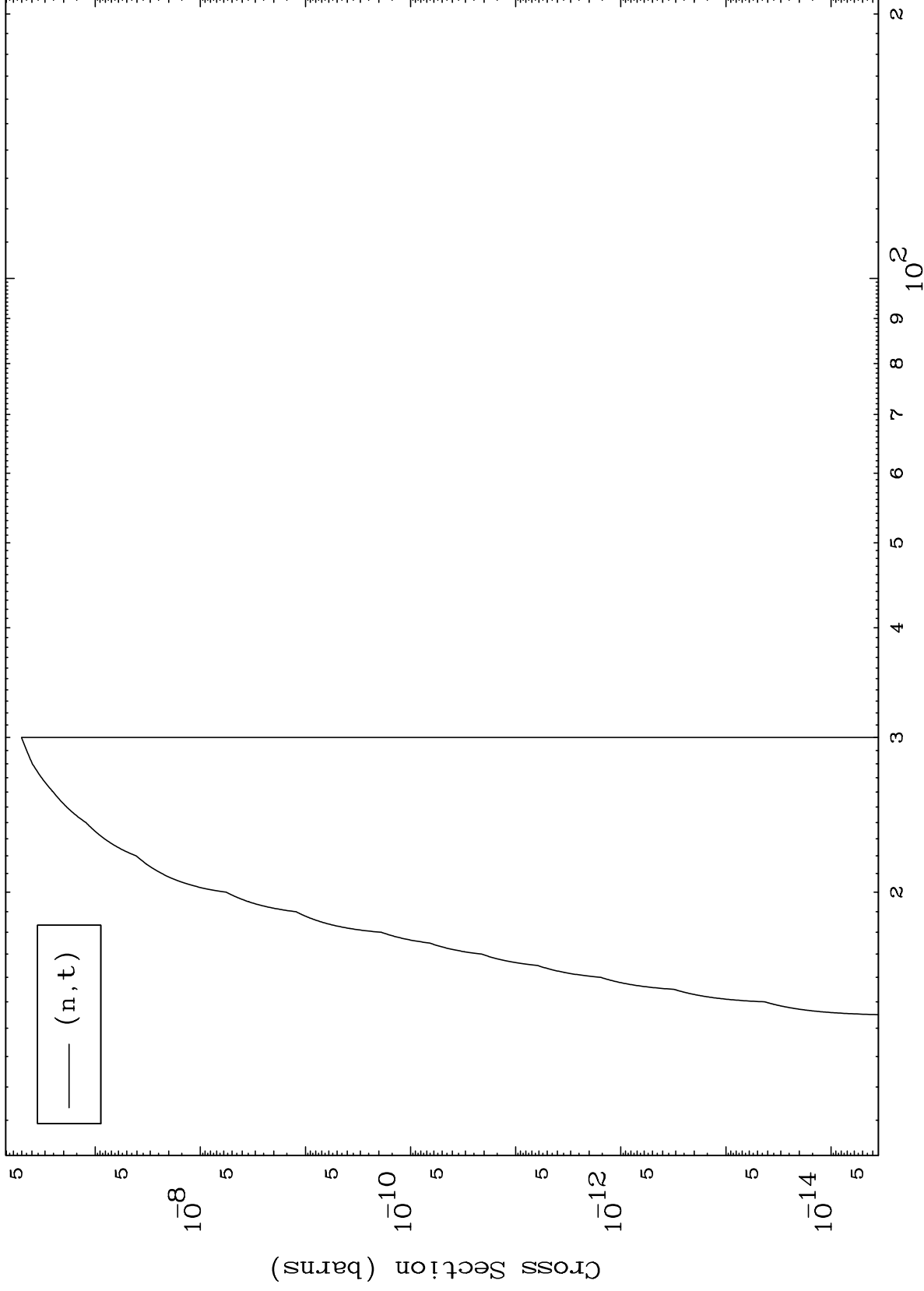
0 Kelvin Cross Sections



Incident Energy (MeV)

77-Ir-186

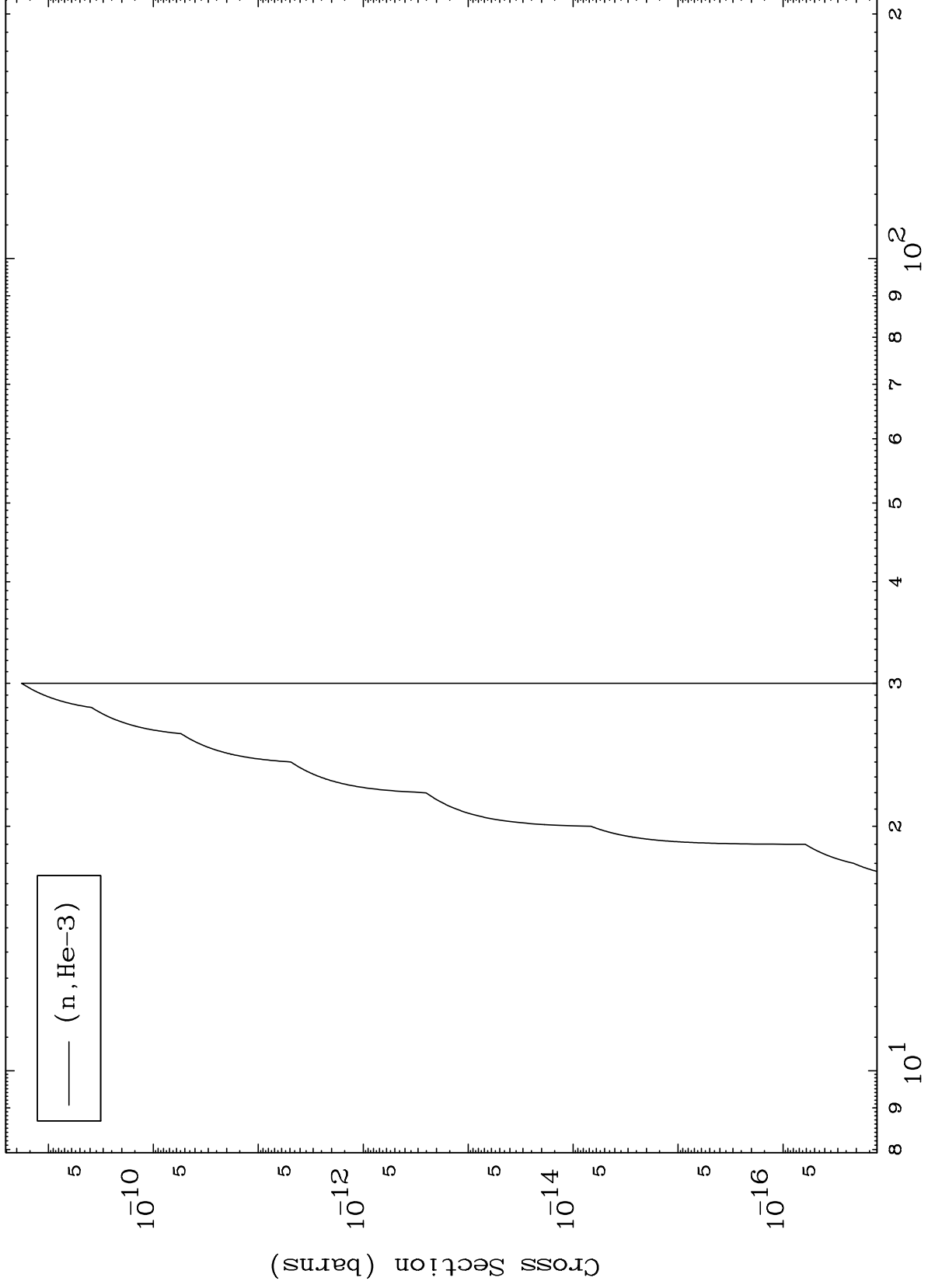
8



MAT 7710

( $\gamma, \text{He}3$ ) Levels  
0 Kelvin Cross Sections

77-Ir-186



10

Incident Energy (MeV)

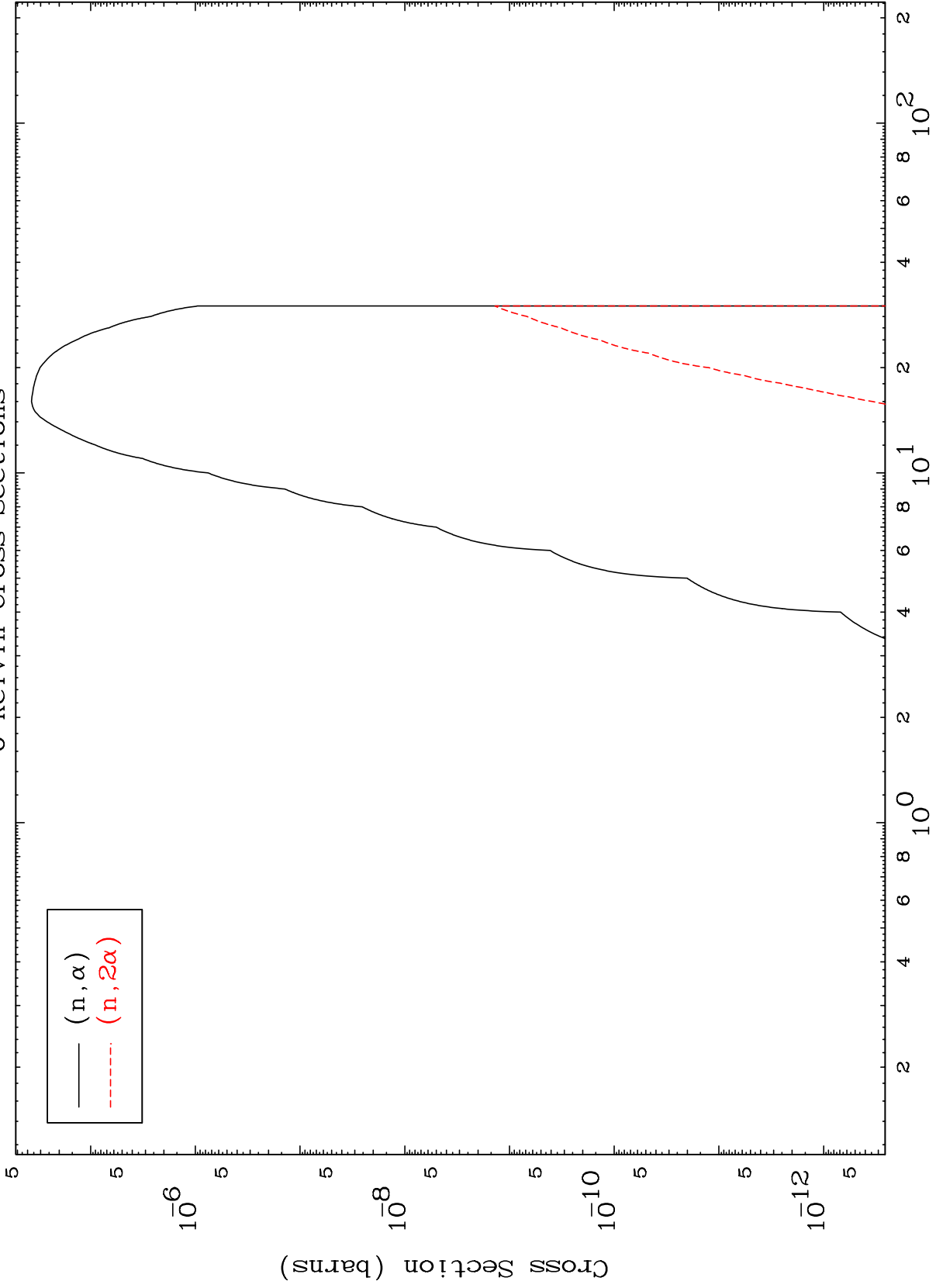
77-Ir-186

MAT 7710

( $\gamma, \alpha$ ) Levels

77-Ir-186

0 Kelvin Cross Sections



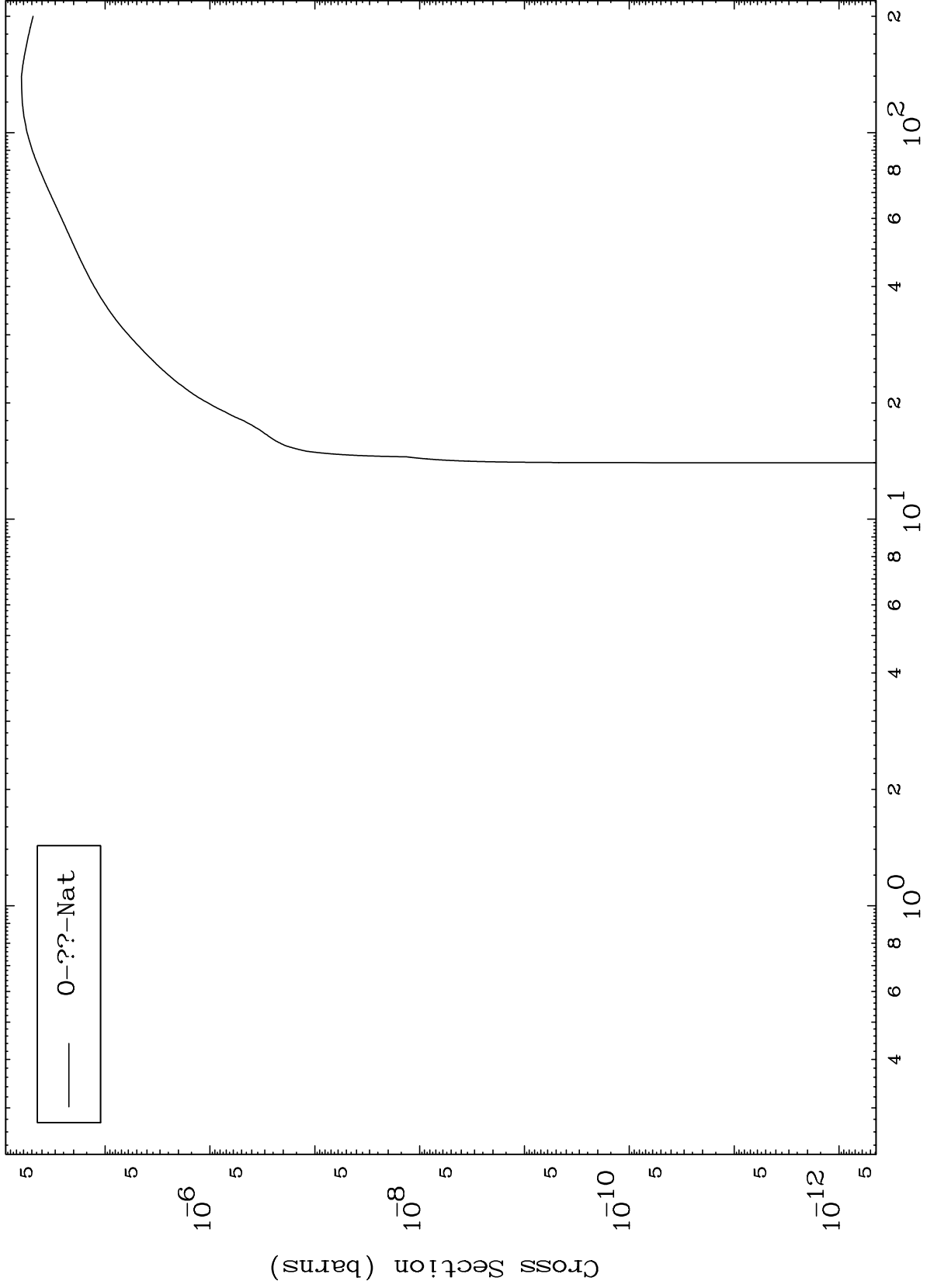
— ( $n, \alpha$ )  
- - - ( $n, 2\alpha$ )

MAT 7710

Fission

<sup>77</sup>Ir-186

Radionuclide Production Cross Section



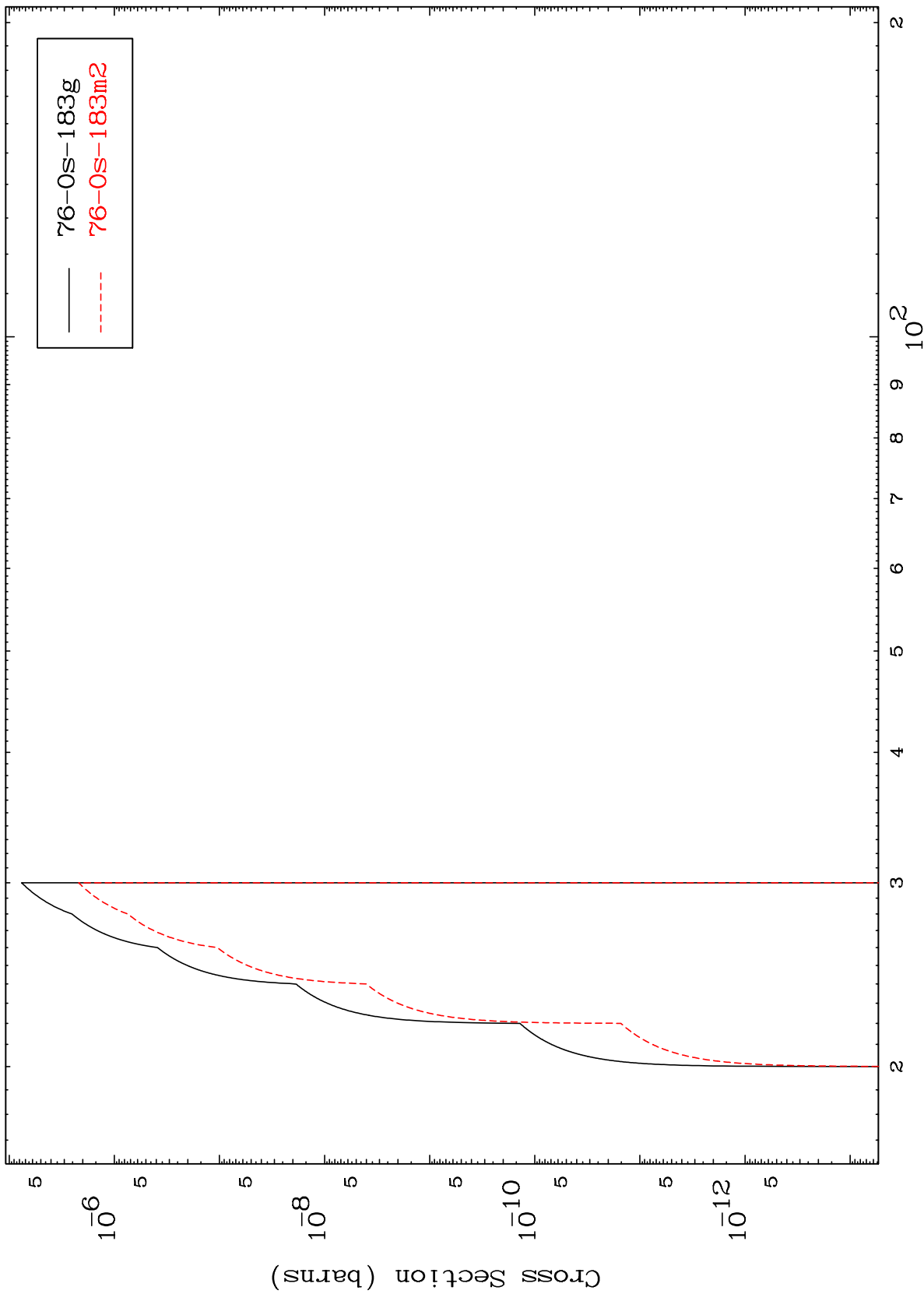
0-??-Nat

MAT 7710

(n,n') d

77-Ir-186

Radionuclide Production Cross Section



13

Incident Energy (MeV)

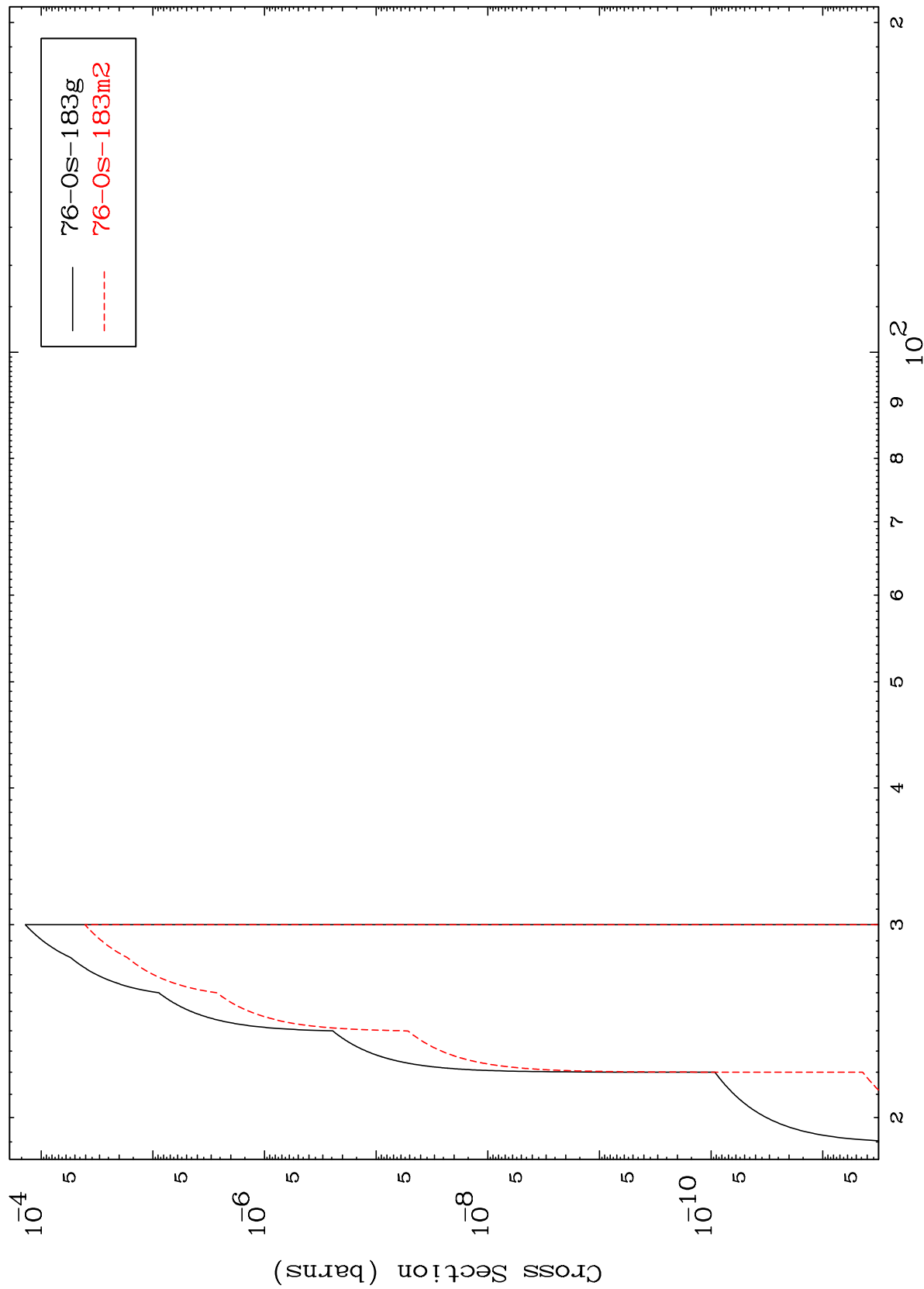
77-Ir-186

MAT 7710

(n,2n) p

77-Ir-186

Radionuclide Production Cross Section



14

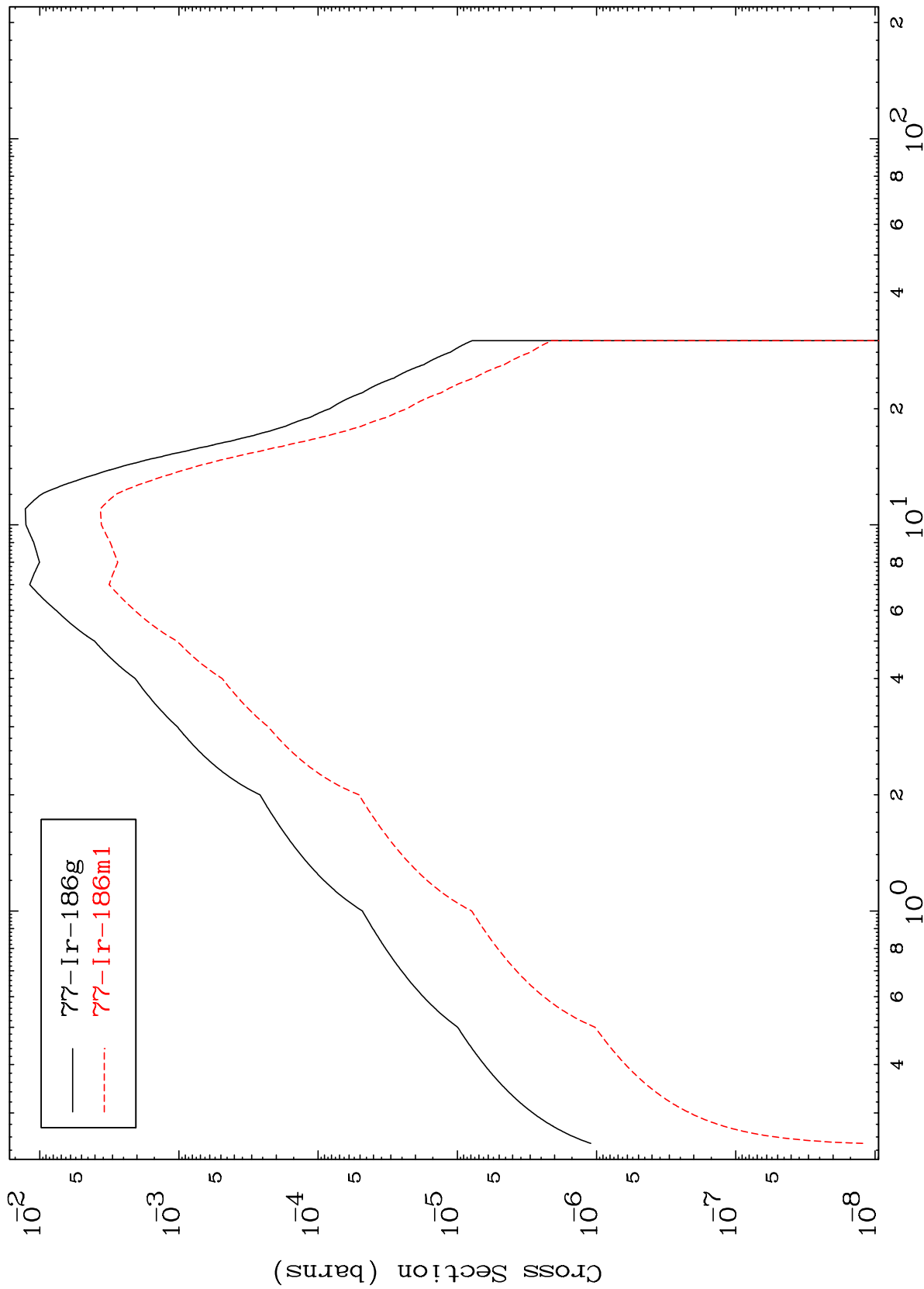
Incident Energy (MeV)

77-Ir-186

MAT 7710

<sup>77</sup>Ir-186

(n,γ)  
Radionuclide Production Cross Section

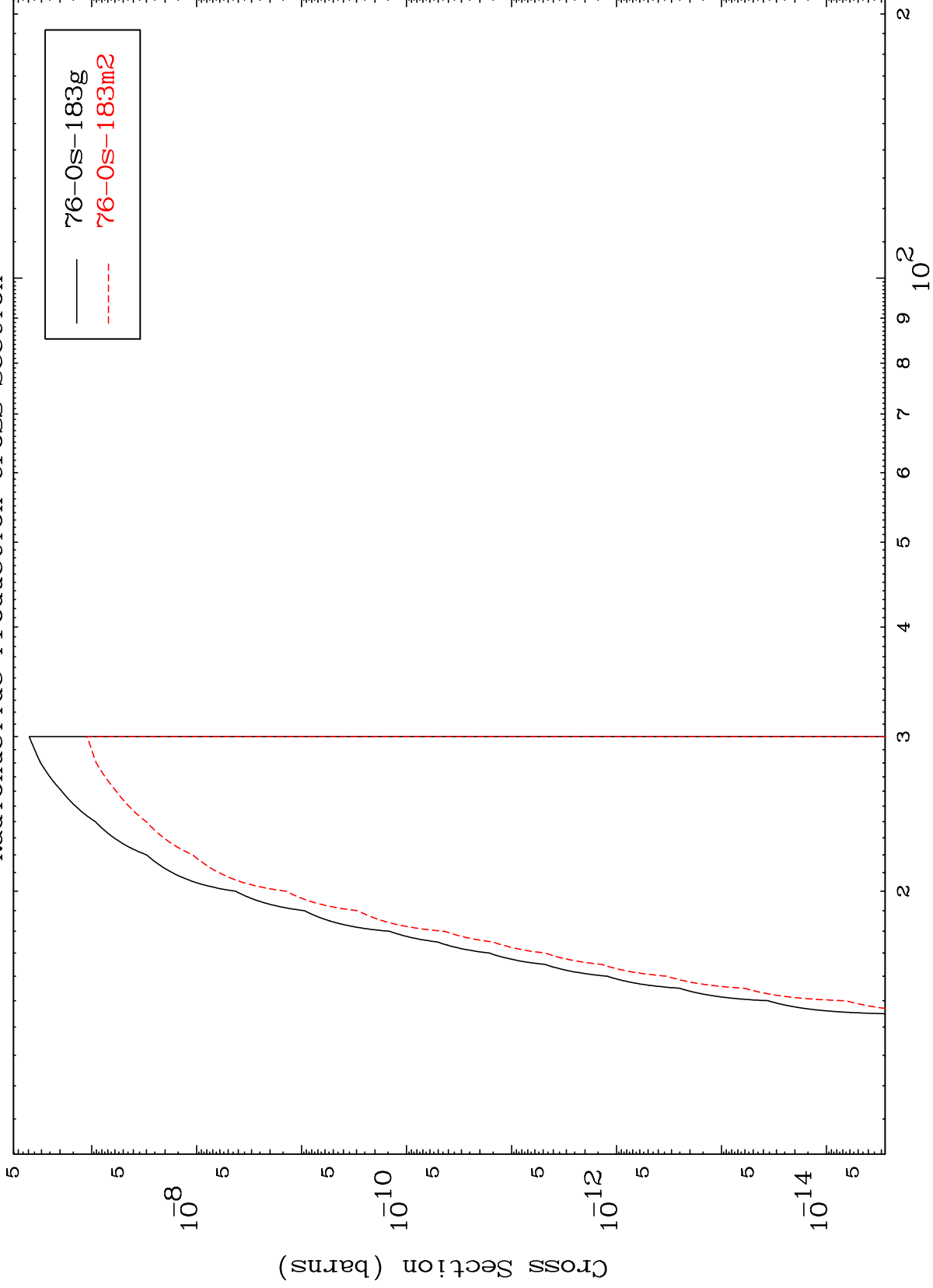


Incident Energy (MeV)

<sup>77</sup>Ir-186

15

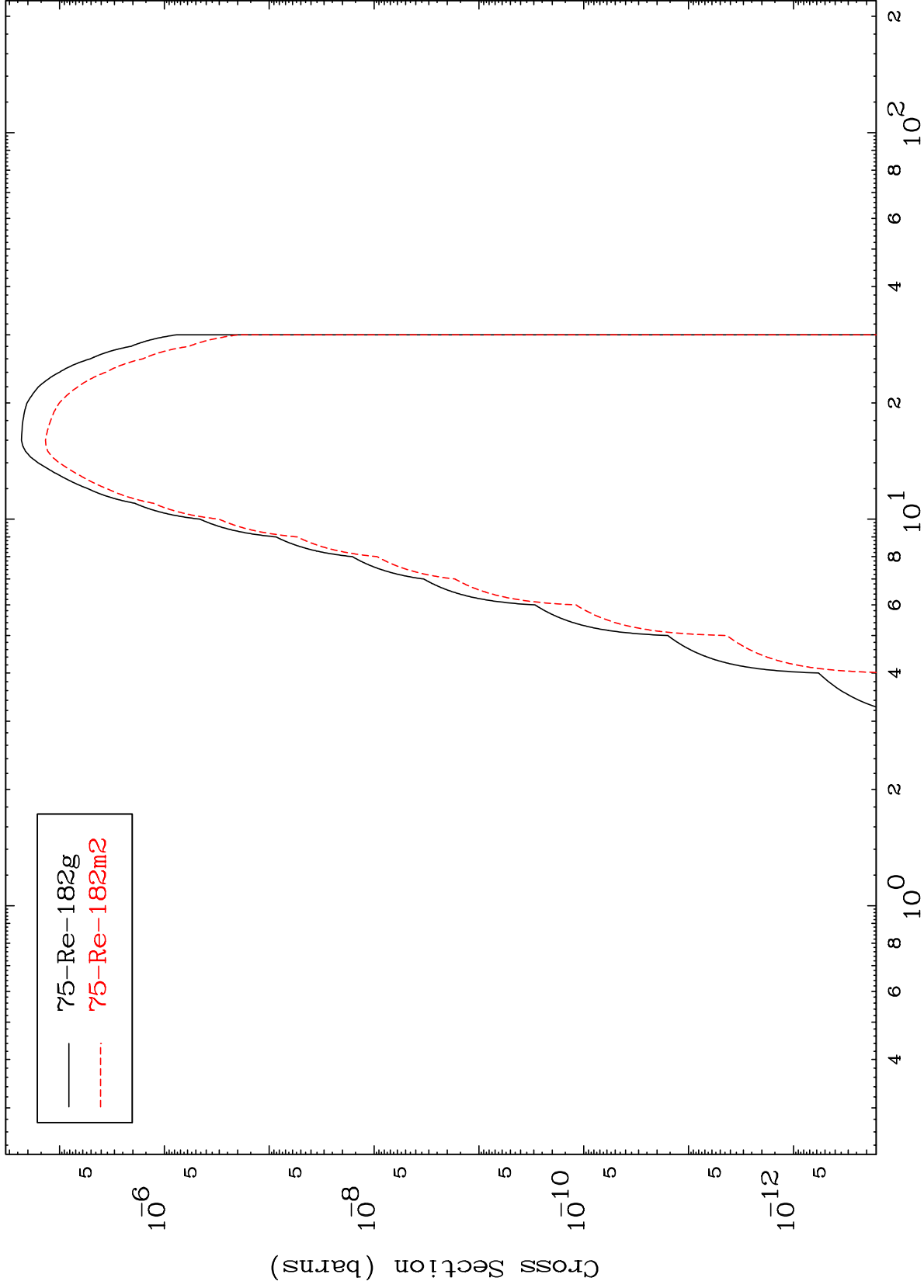
Radionuclide Production Cross Section



MAT 7710

77-Ir-186

Radionuclide Production Cross Section



17

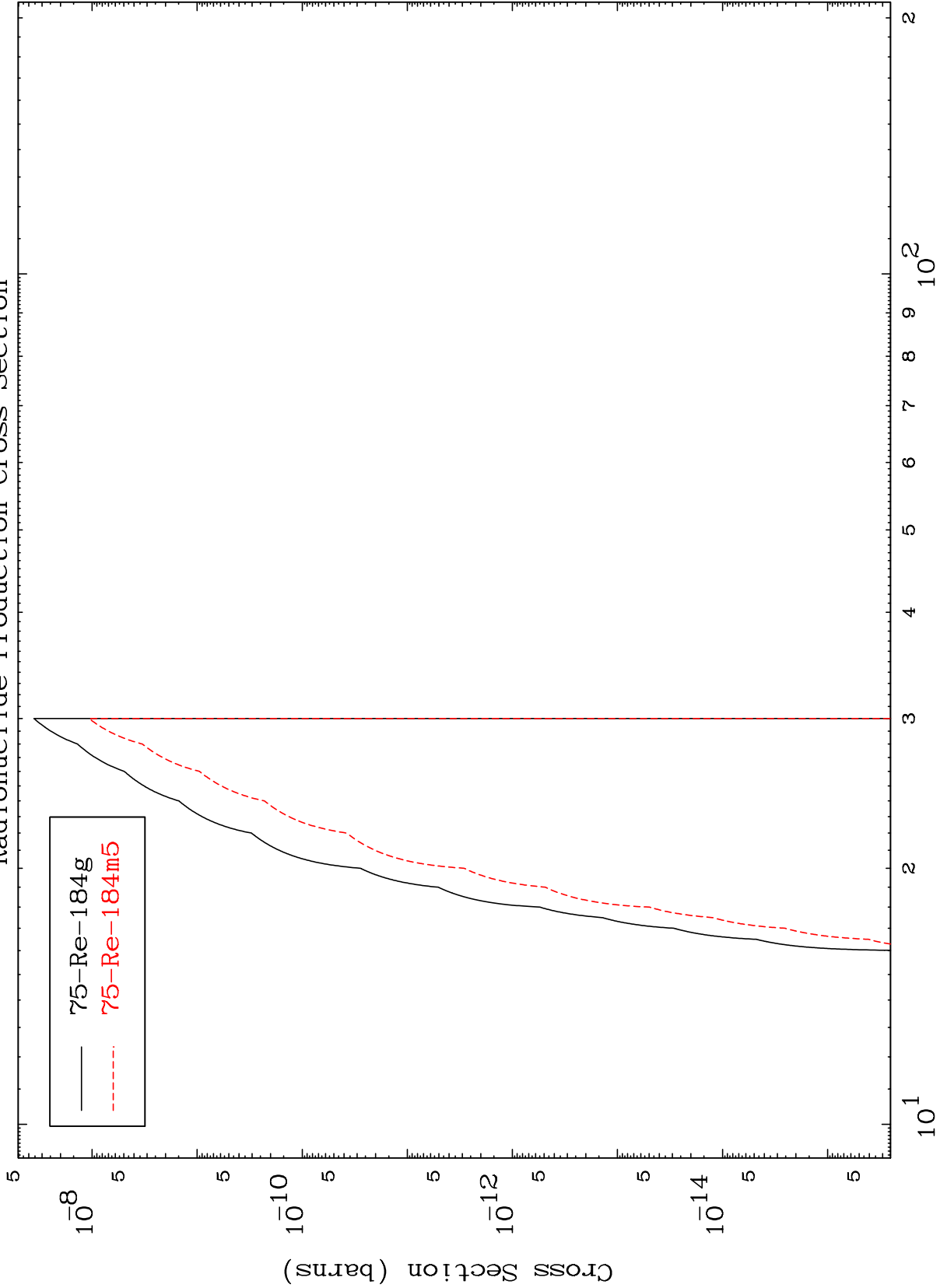
77-Ir-186

MAT 7710

(n,2p)

<sup>77</sup>Ir-186

Radionuclide Production Cross Section



Incident Energy (MeV)

<sup>77</sup>Ir-186