

Program EVALPLOT  
(Version 2021-1)

by

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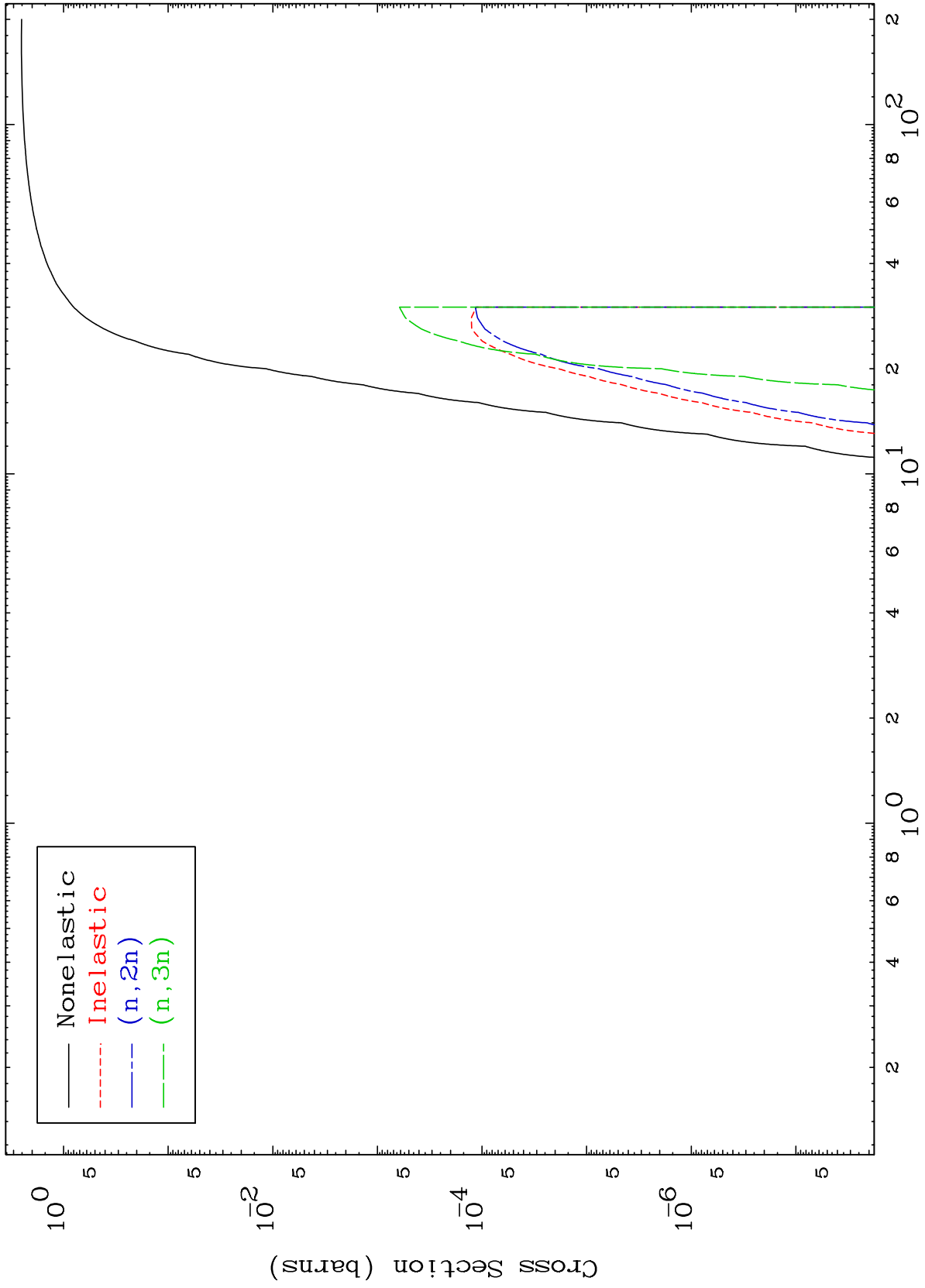
Press Mouse Button to Start

MAT 8916

He-3 Major

89-Ac-222

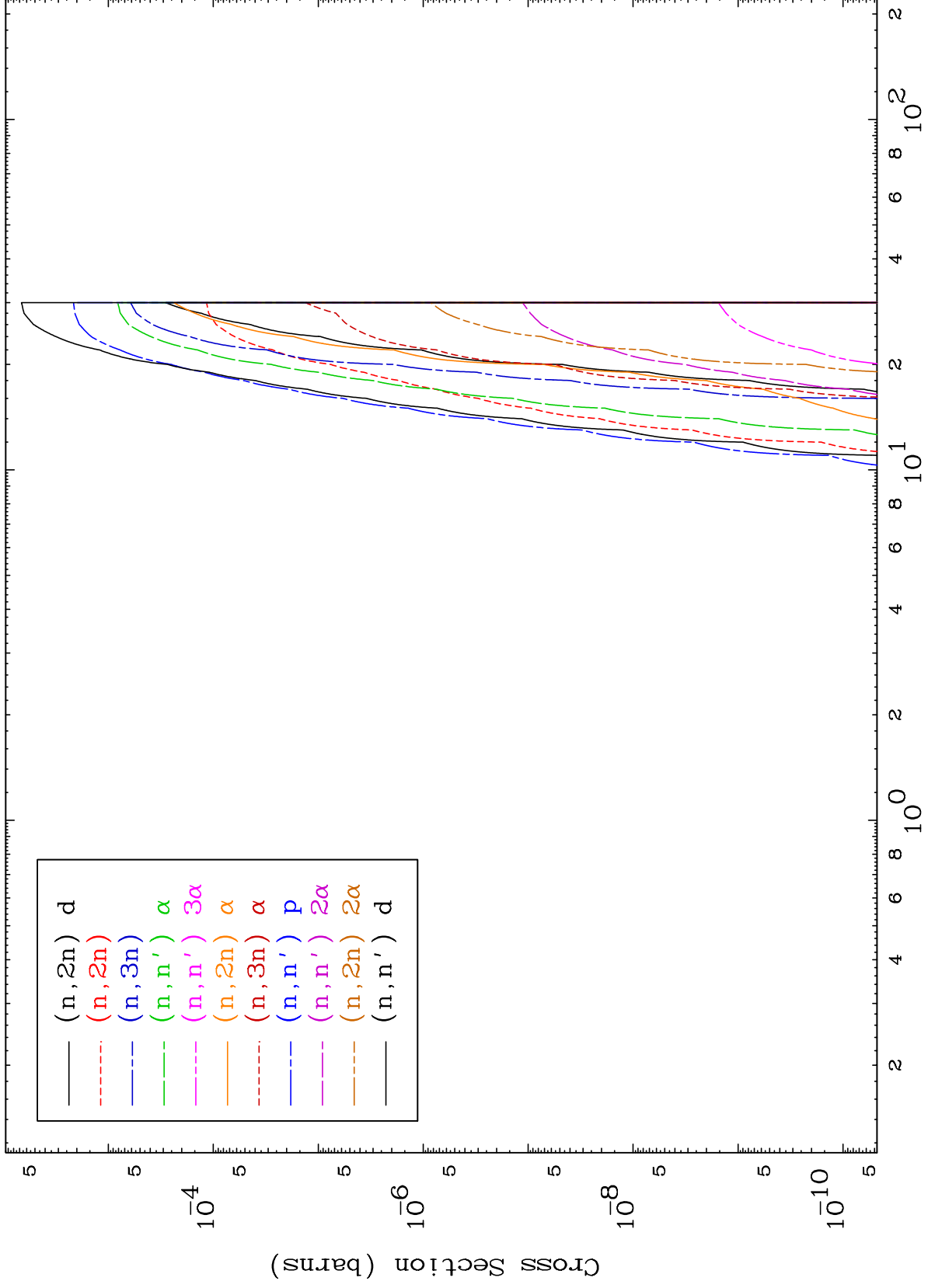
0 Kelvin Cross Sections

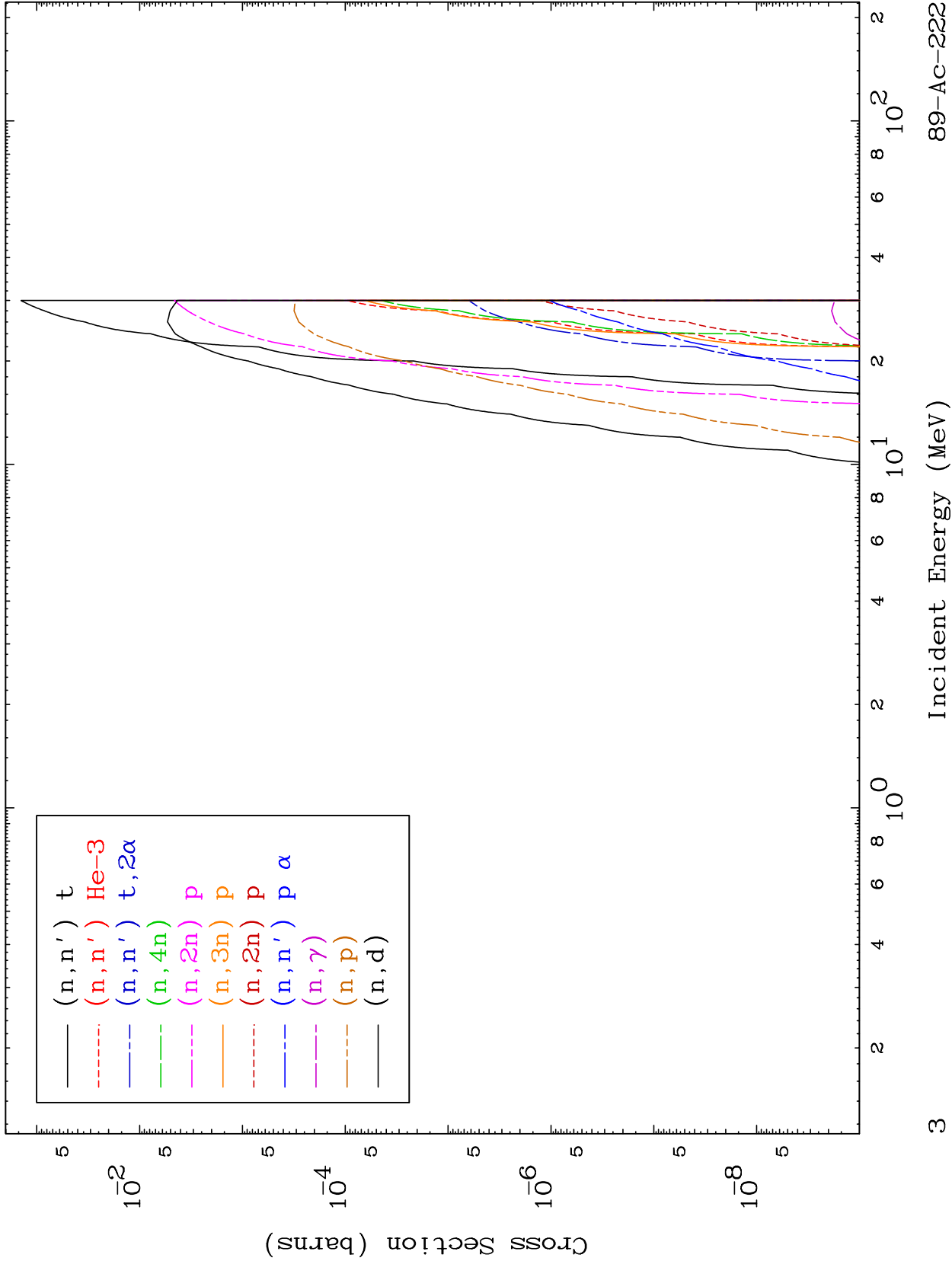


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He-3 Neutron Absorption  
0 Kelvin Cross Sections

89-Ac-222

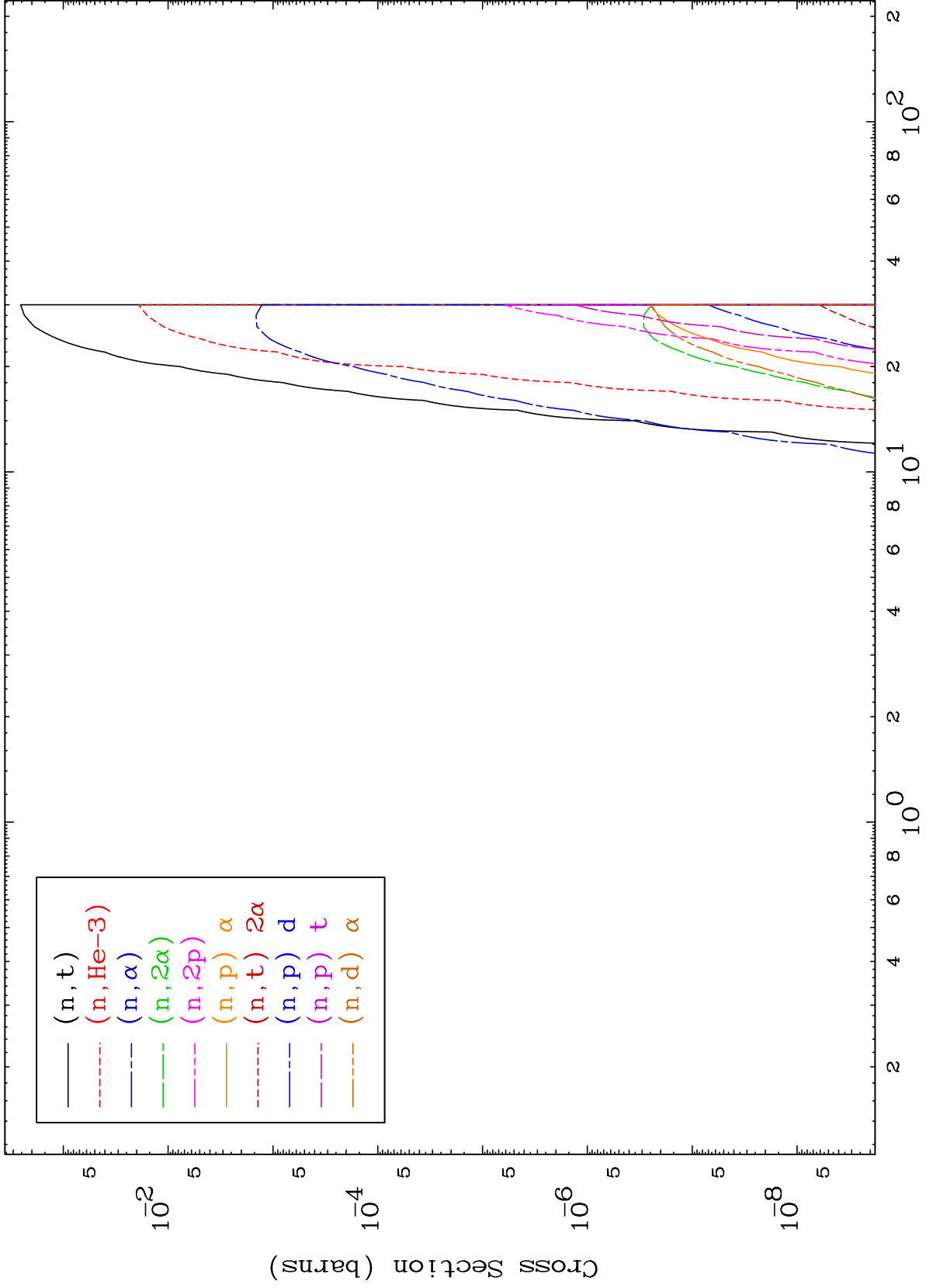


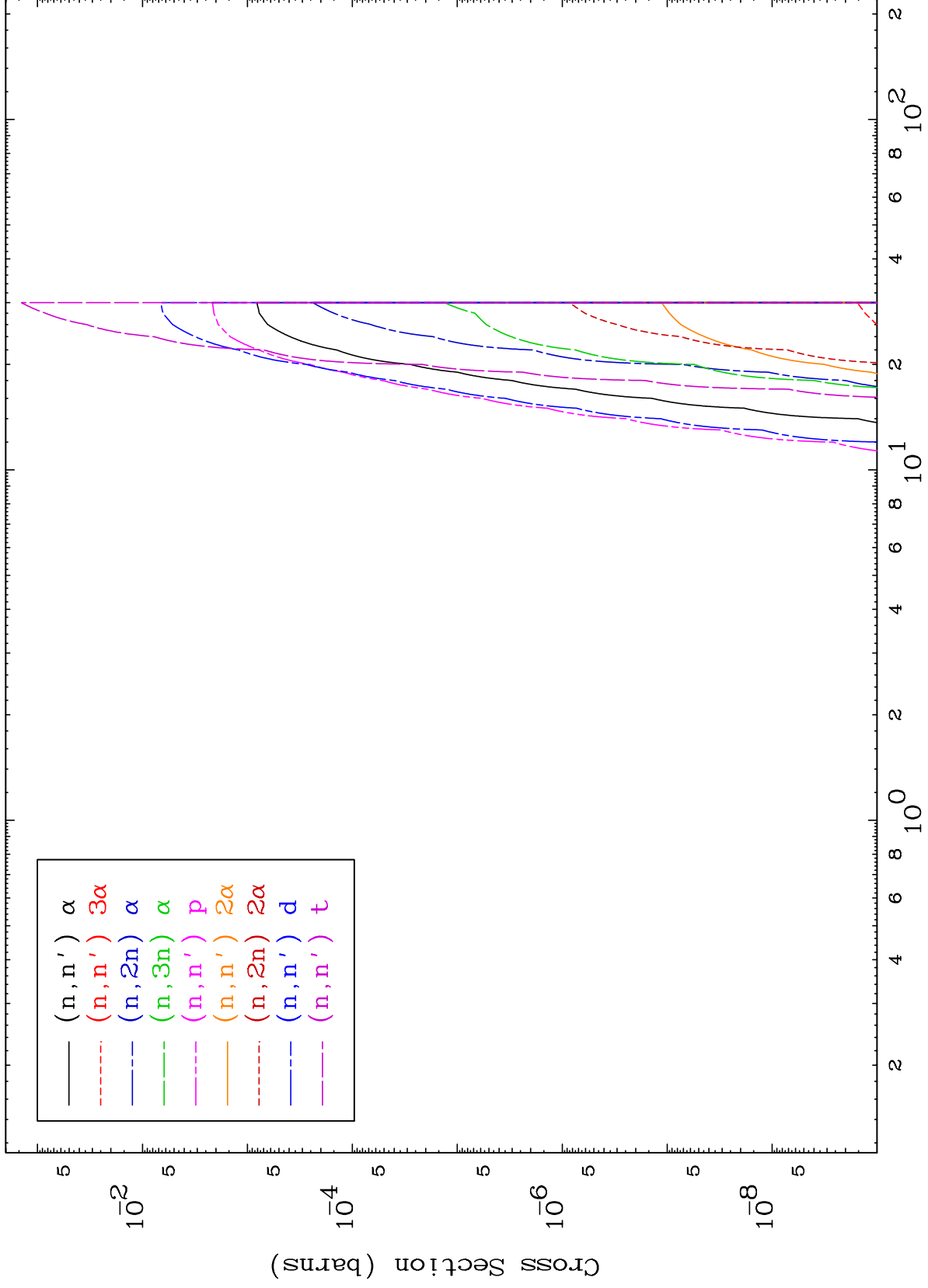


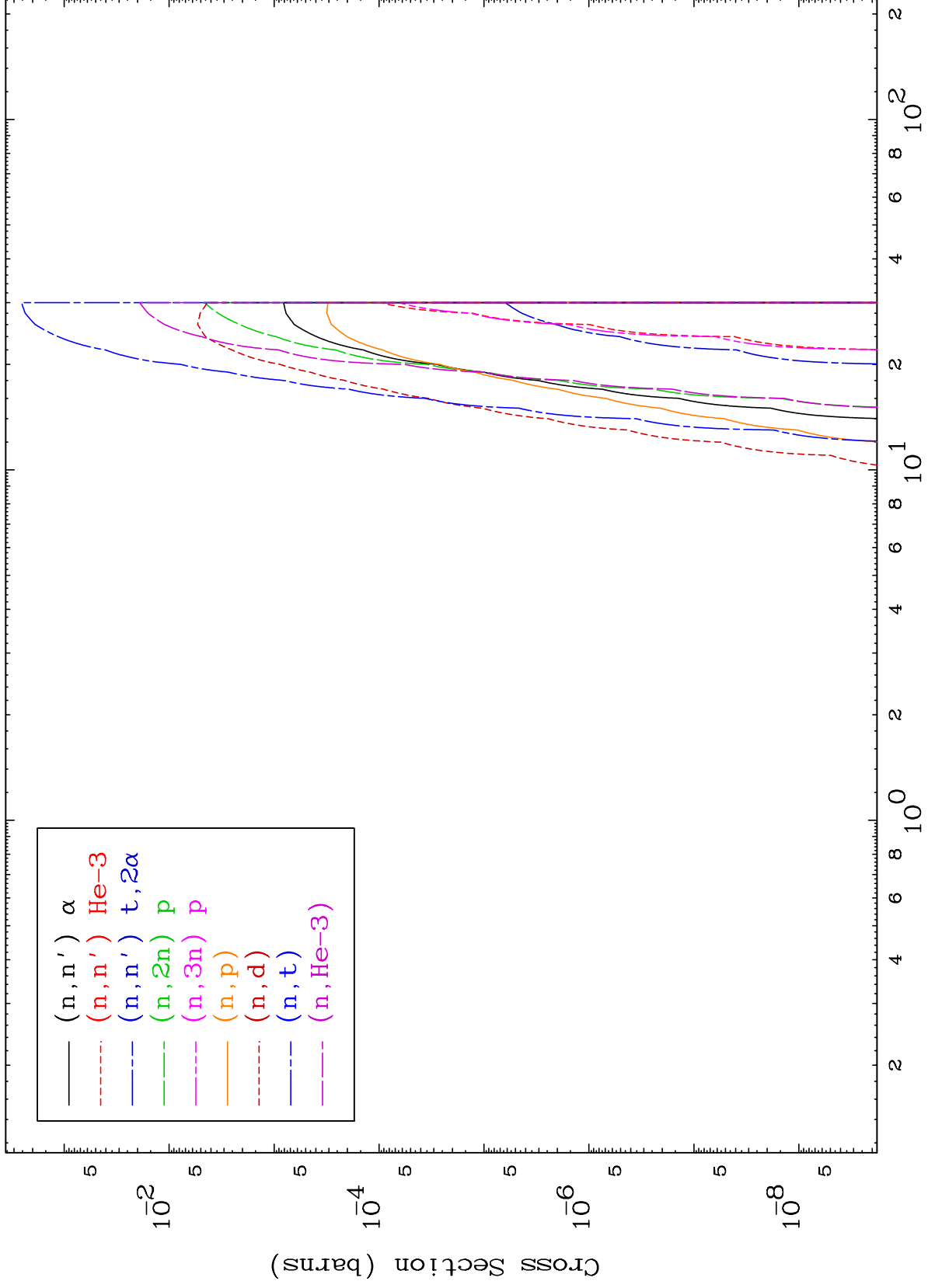
MAT 8916

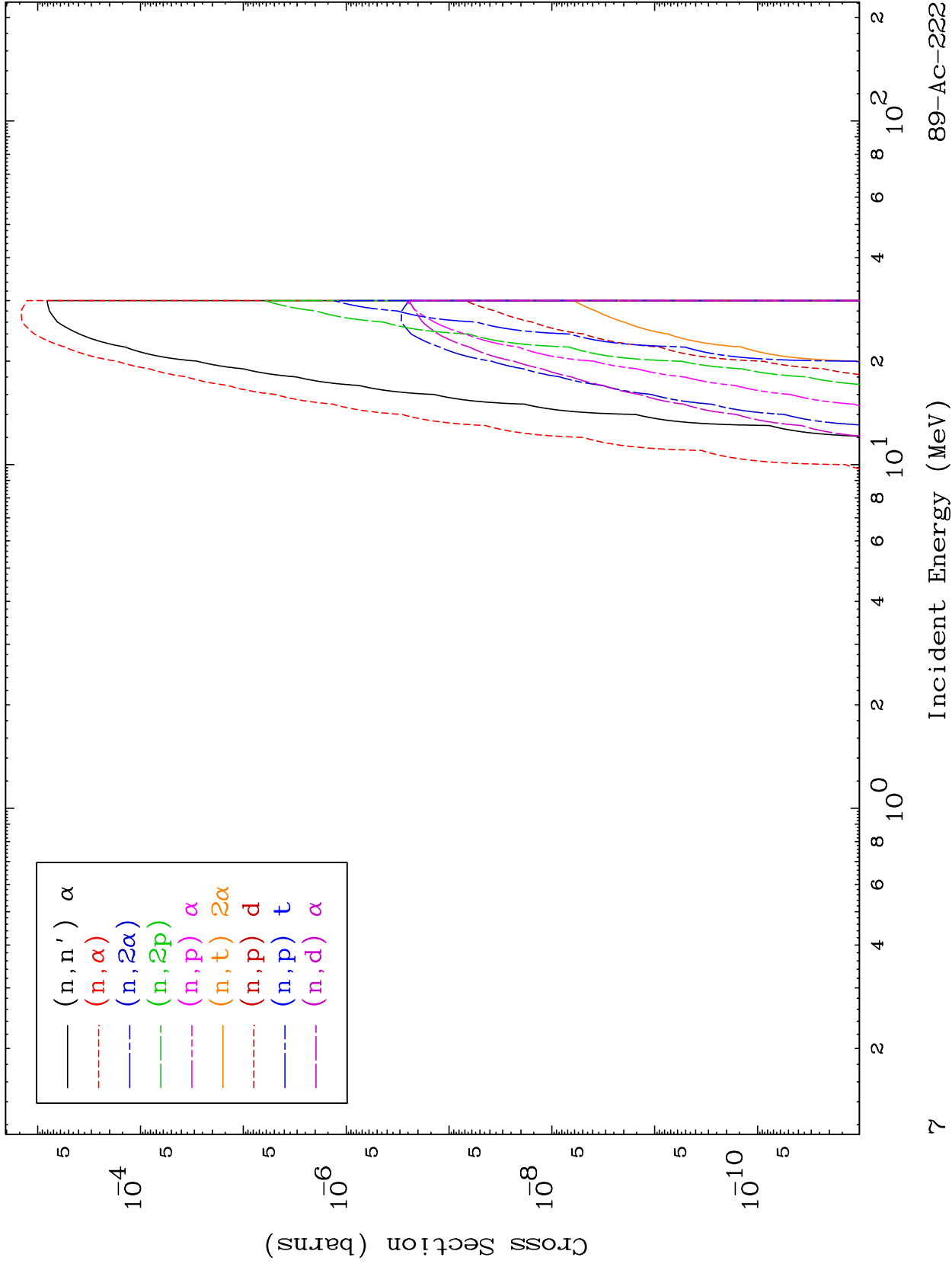
He-3 Neutron Absorption  
0 Kelvin Cross Sections

89-Ac-222





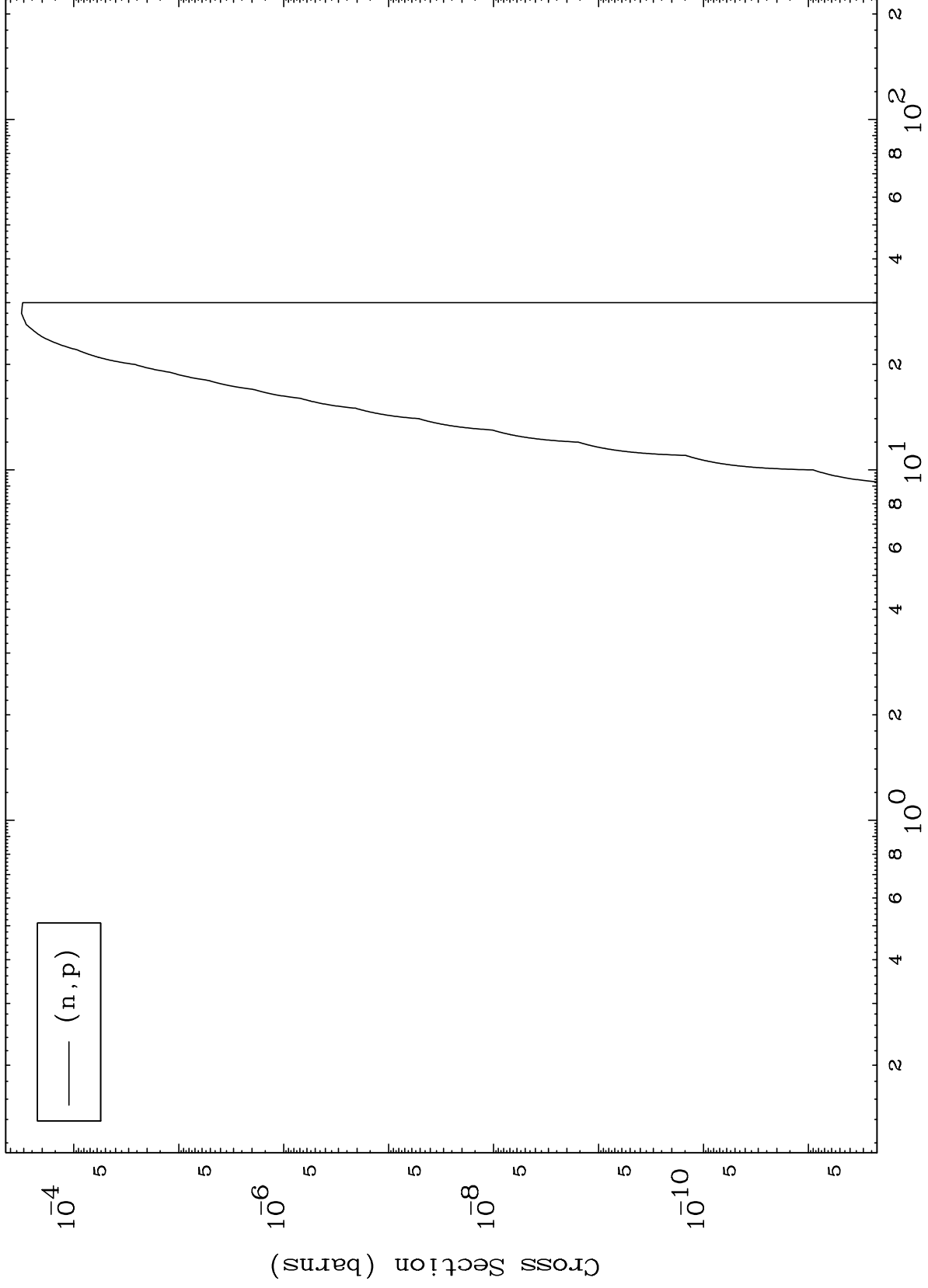




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(He-3,p) Levels  
0 Kelvin Cross Sections

89-Ac-222

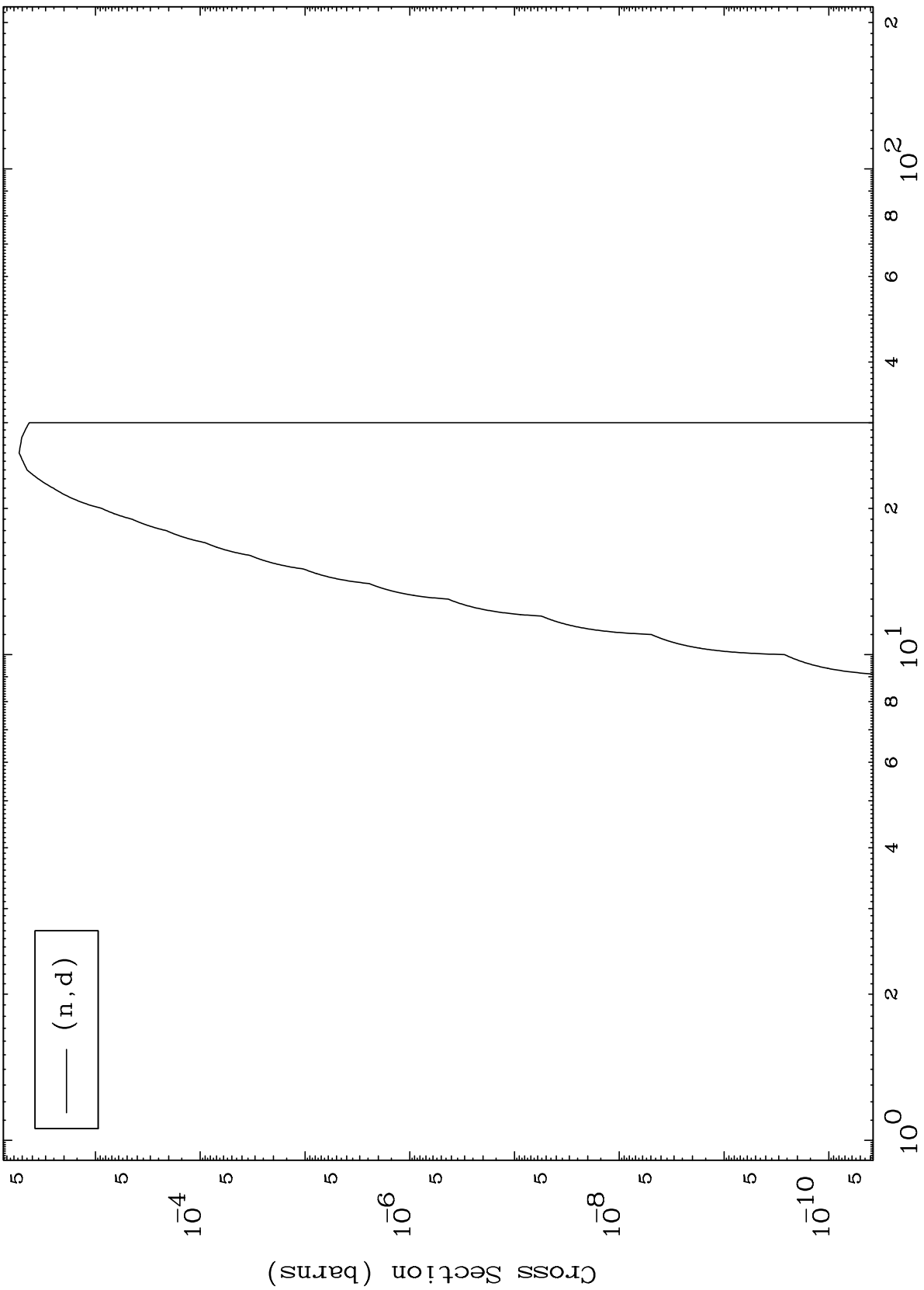


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(He-3,d) Levels

89-Ac-222

0 Kelvin Cross Sections



Incident Energy (MeV)

89-Ac-222

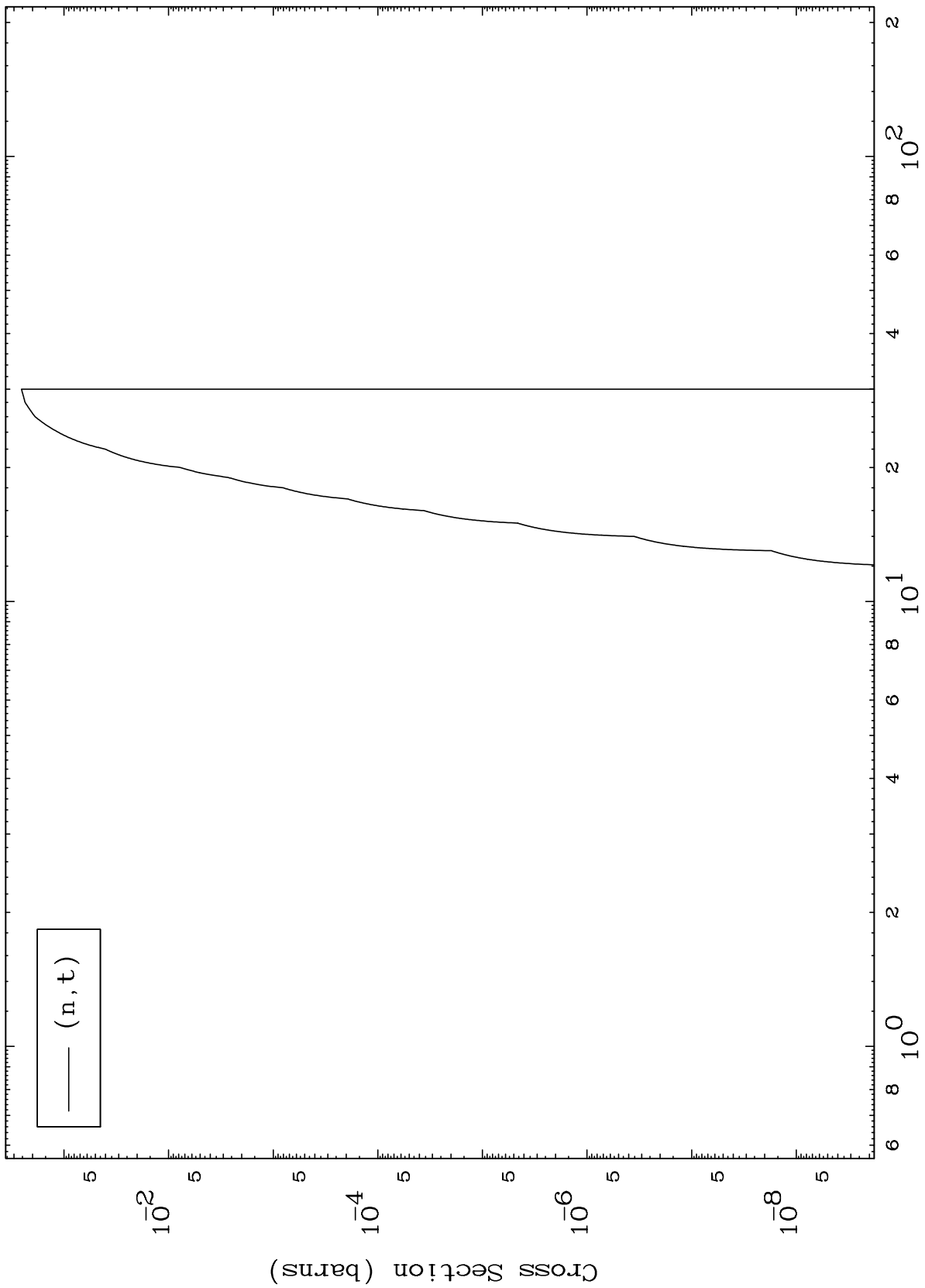
9

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(He-3,t) Levels

89-Ac-222

0 Kelvin Cross Sections



10

Incident Energy (MeV)

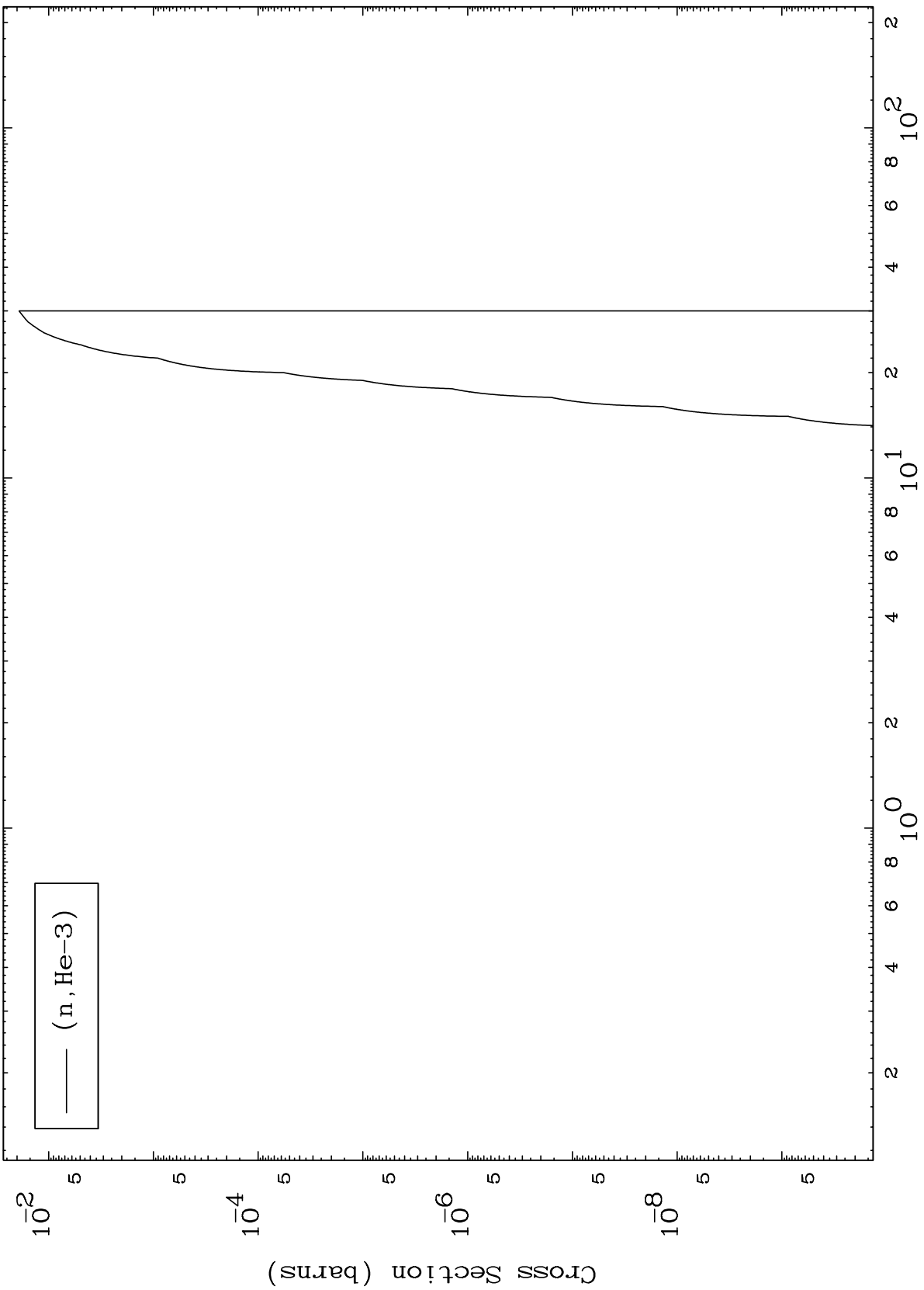
89-Ac-222

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(He-3, He3) Levels

89-Ac-222

0 Kelvin Cross Sections

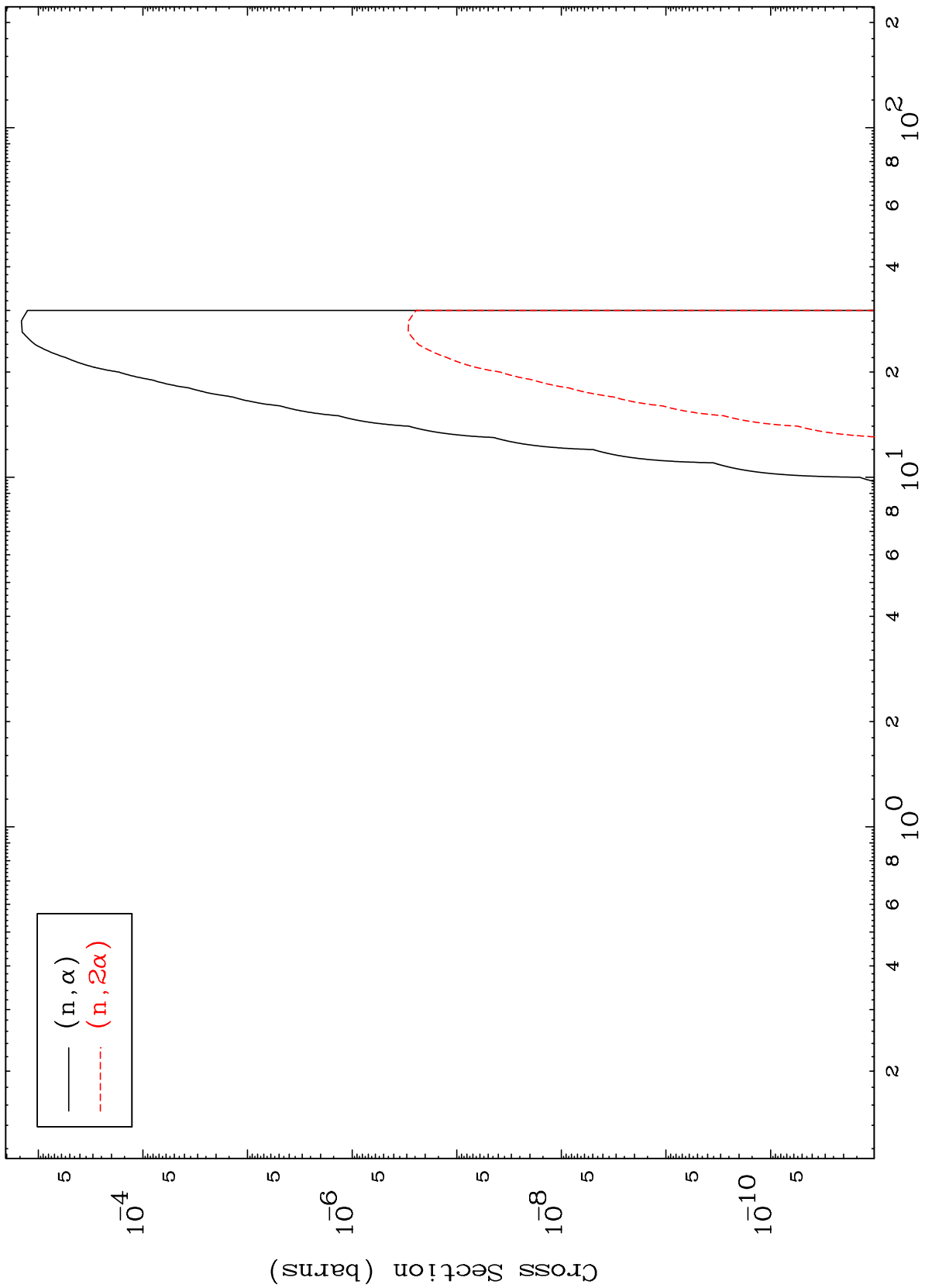


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(He-3,  $\alpha$ ) Levels

89-Ac-222

0 Kelvin Cross Sections

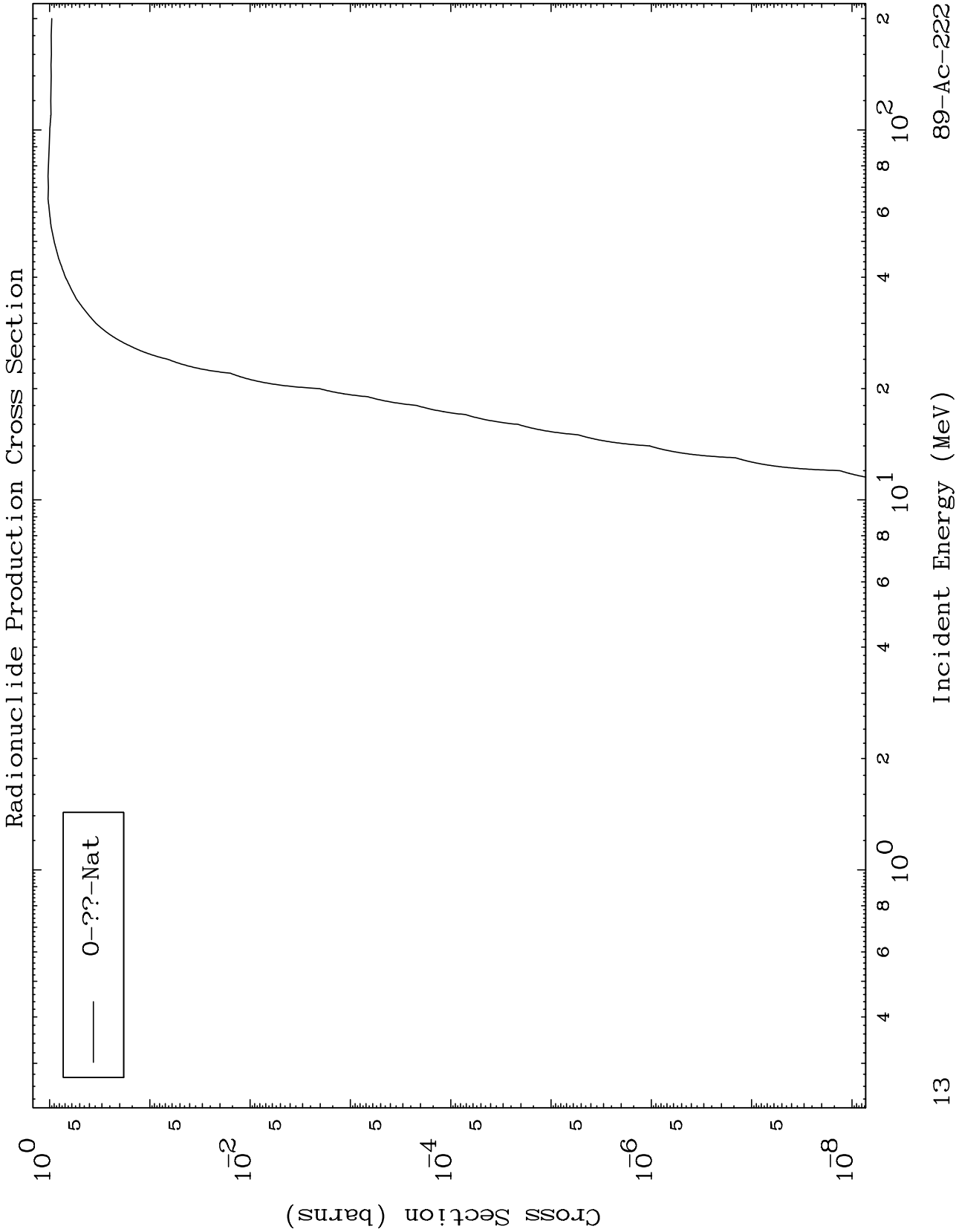


—  $(n, \alpha)$   
- - -  $(n, 2\alpha)$

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Fission

89-Ac-222

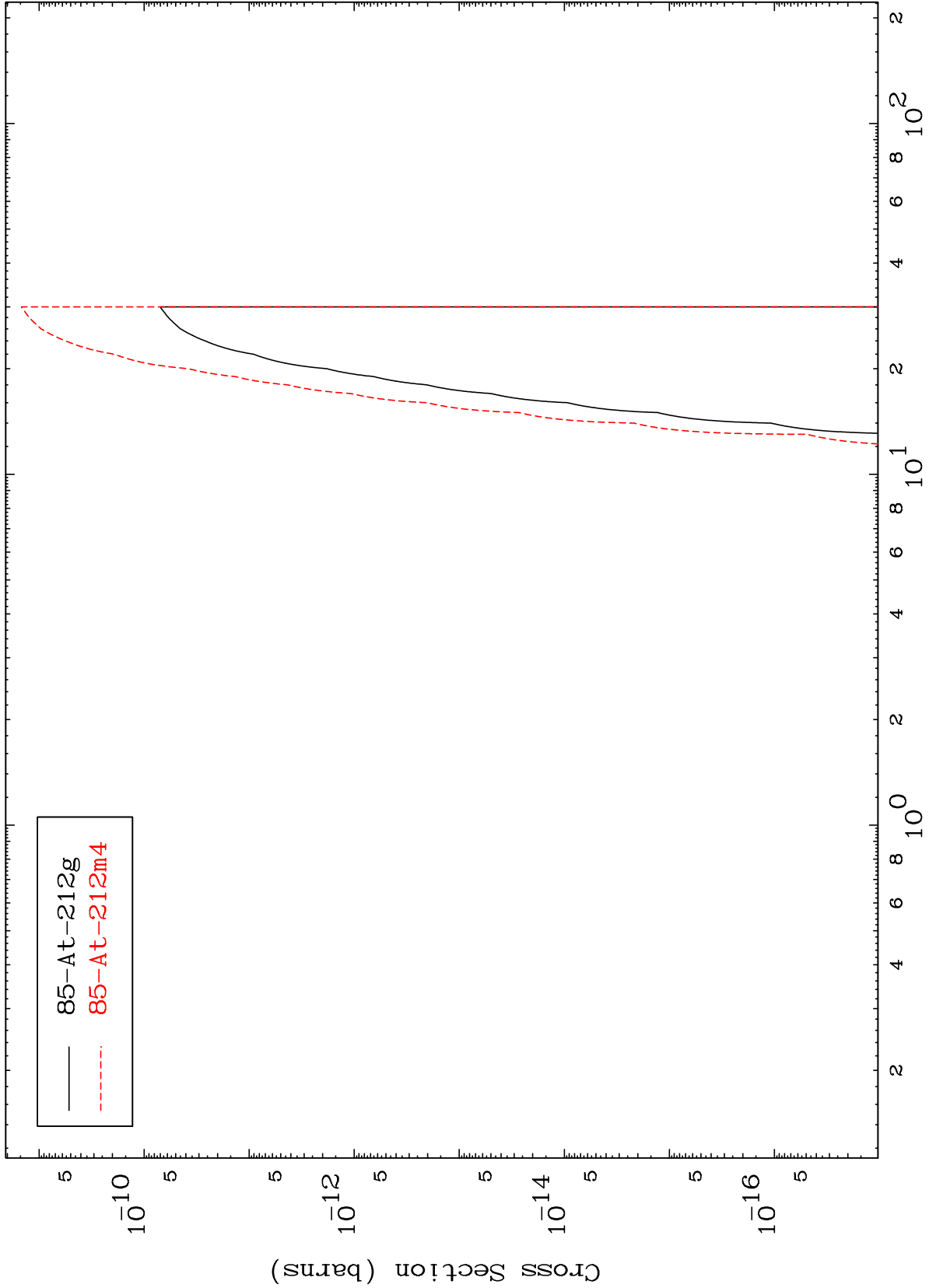


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(n,n') 3 $\alpha$

89-Ac-222

Radionuclide Production Cross Section



85-At-212g  
85-At-212m4

14

Incident Energy (MeV)

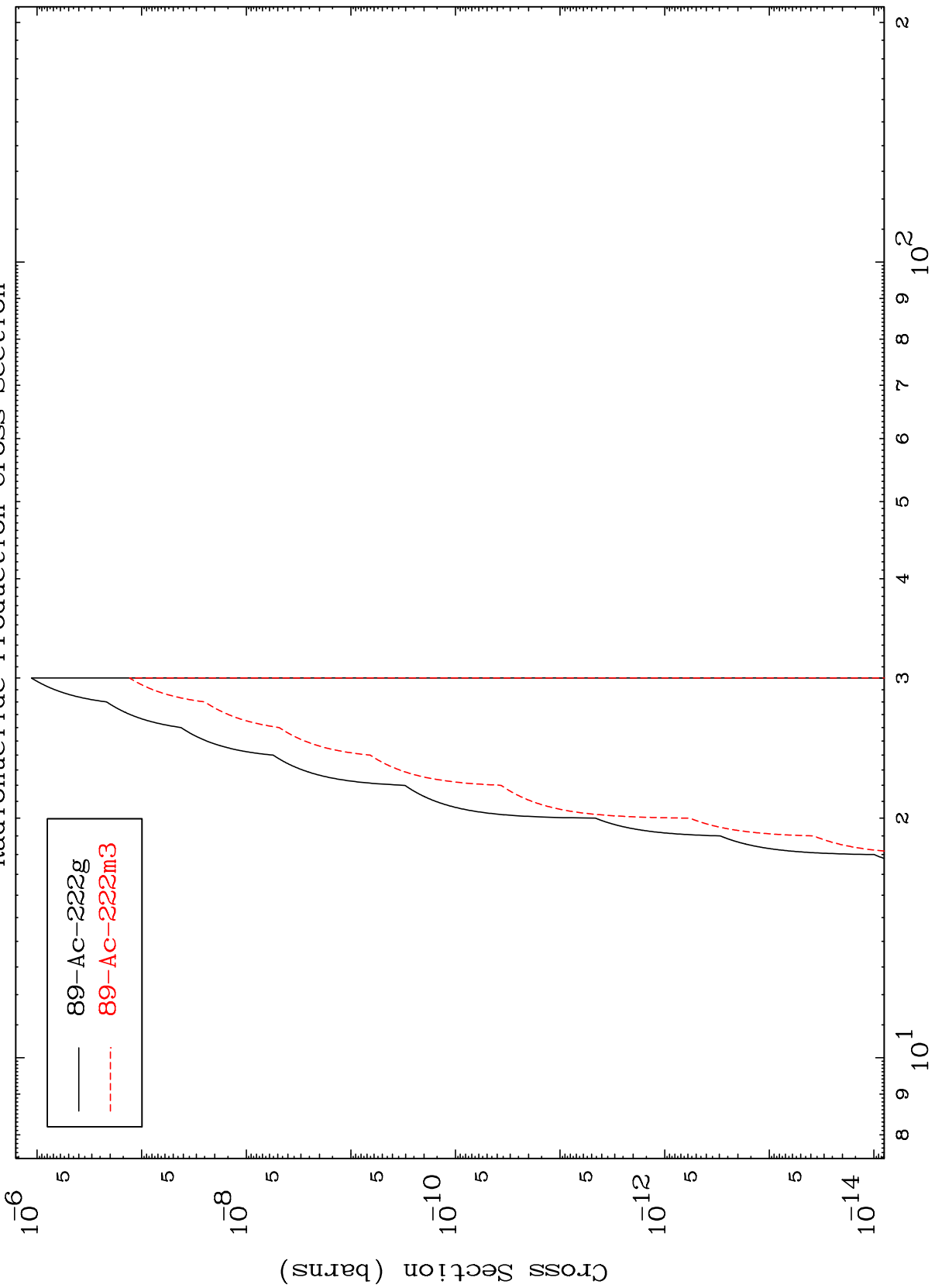
89-Ac-222

MAT 8916

(n,2n) p

89-Ac-222

Radionuclide Production Cross Section



89-Ac-222g  
89-Ac-222m3

15

Incident Energy (MeV)

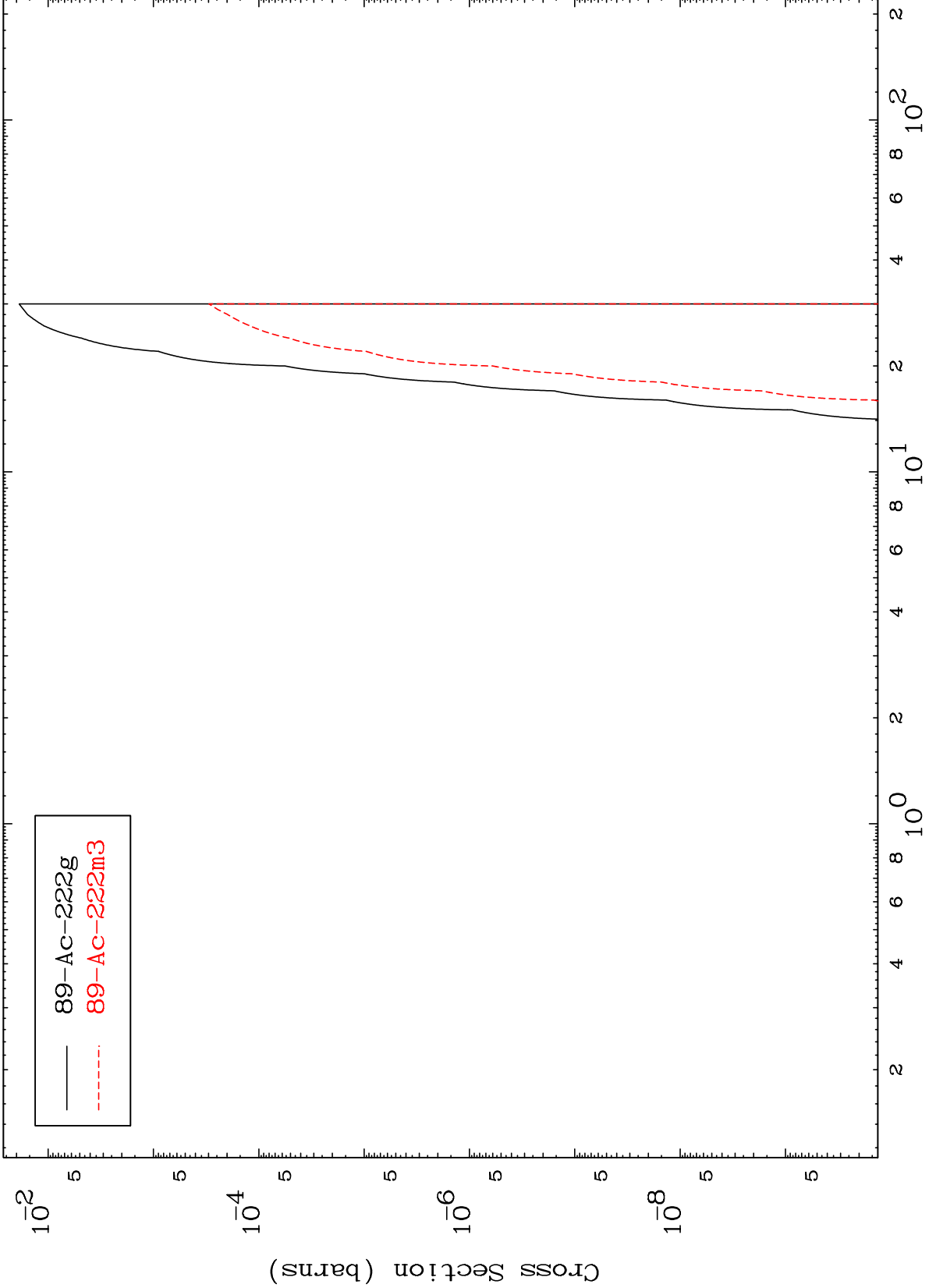
89-Ac-222

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(n,He-3)

89-Ac-222

Radionuclide Production Cross Section

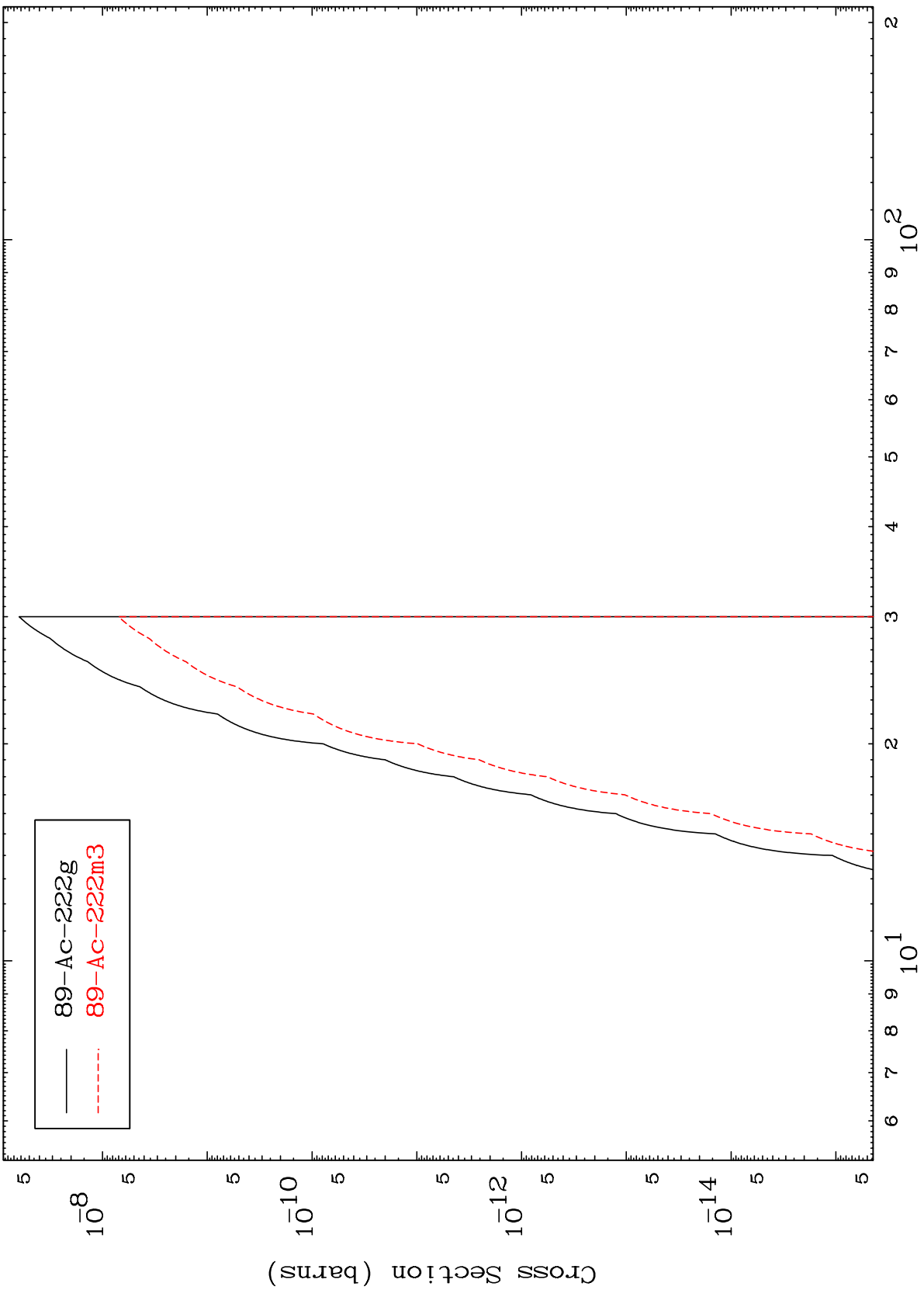


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(n,p) d

89-Ac-222

Radionuclide Production Cross Section



17

Incident Energy (MeV)

89-Ac-222