

Program EVALPLOT  
(Version 2021-1)

by

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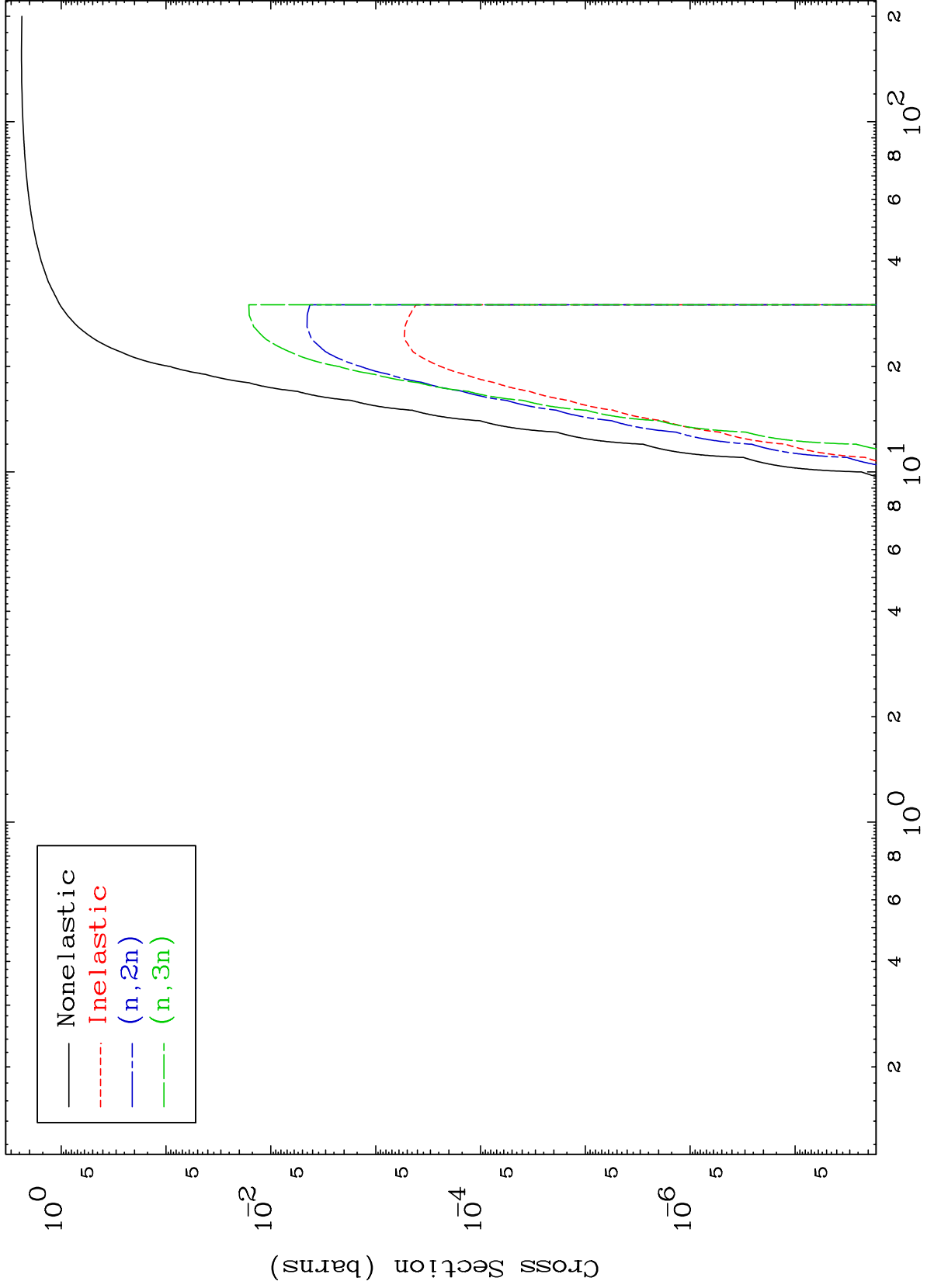
Press Mouse Button to Start

MAT 7734

He-3 Major

0 Kelvin Cross Sections

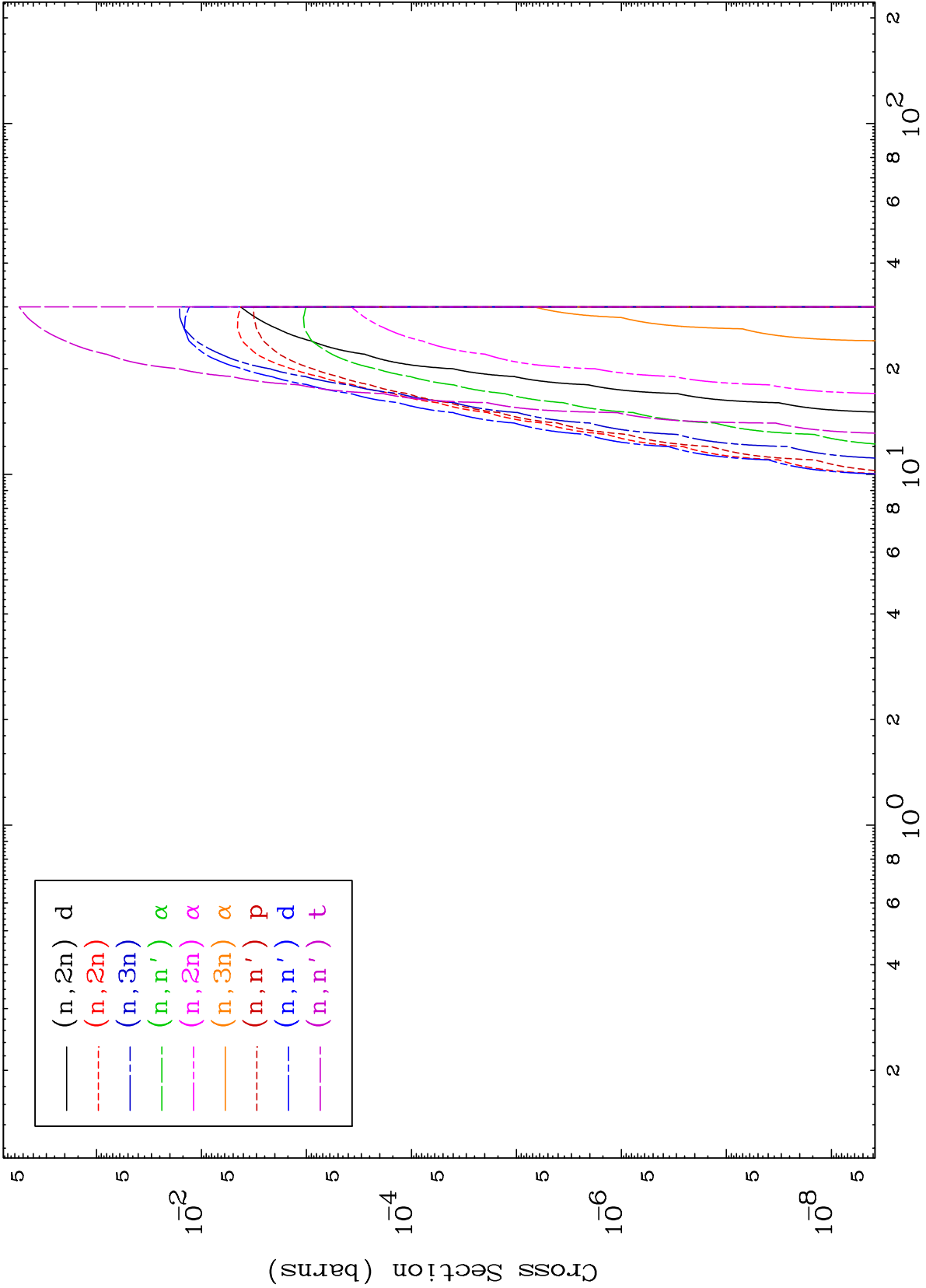
77-Ir-194



MAT 7734

He-3 Neutron Absorption  
0 Kelvin Cross Sections

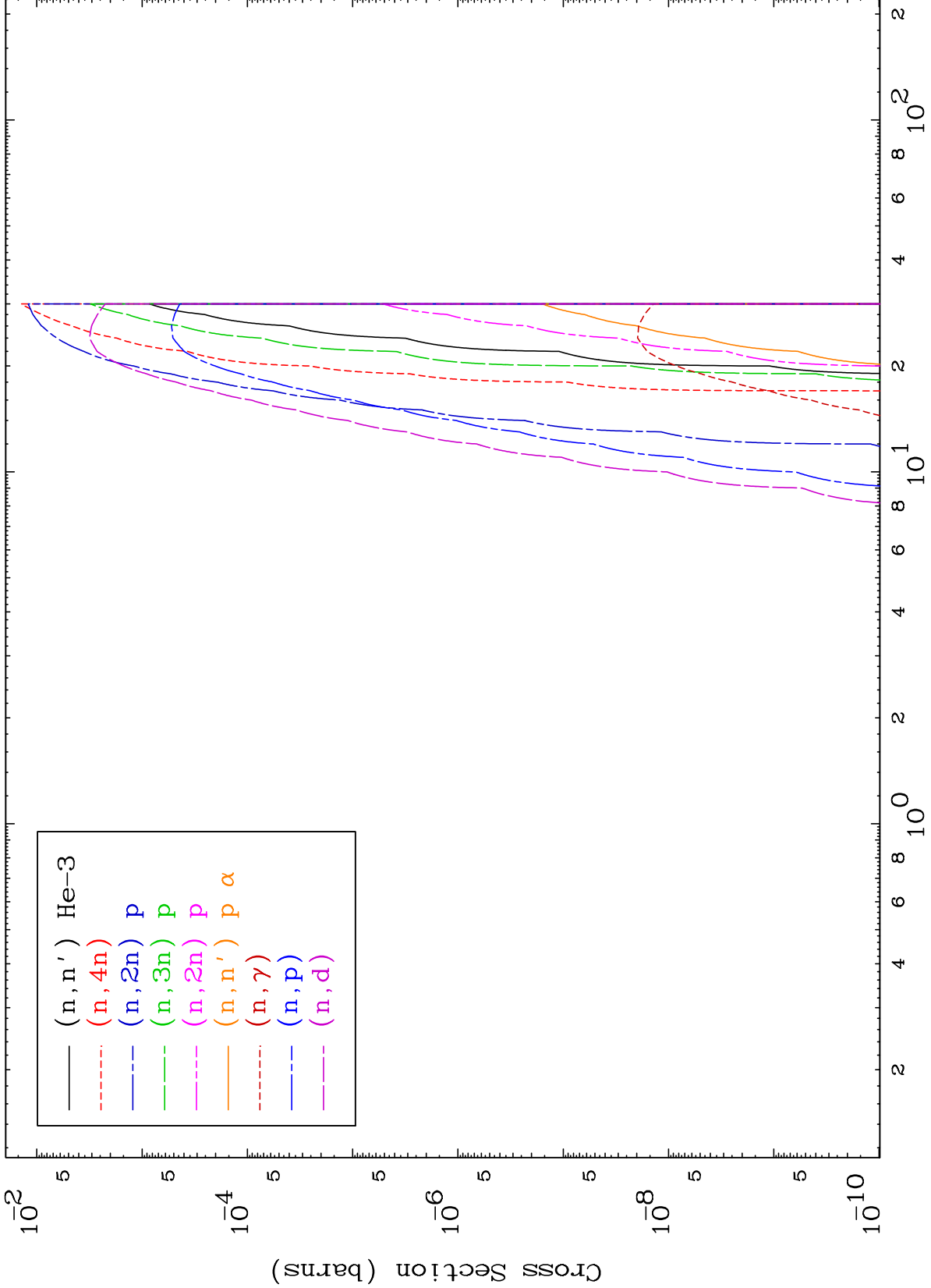
77-Ir-194



MAT 7734

He-3 Neutron Absorption  
0 Kelvin Cross Sections

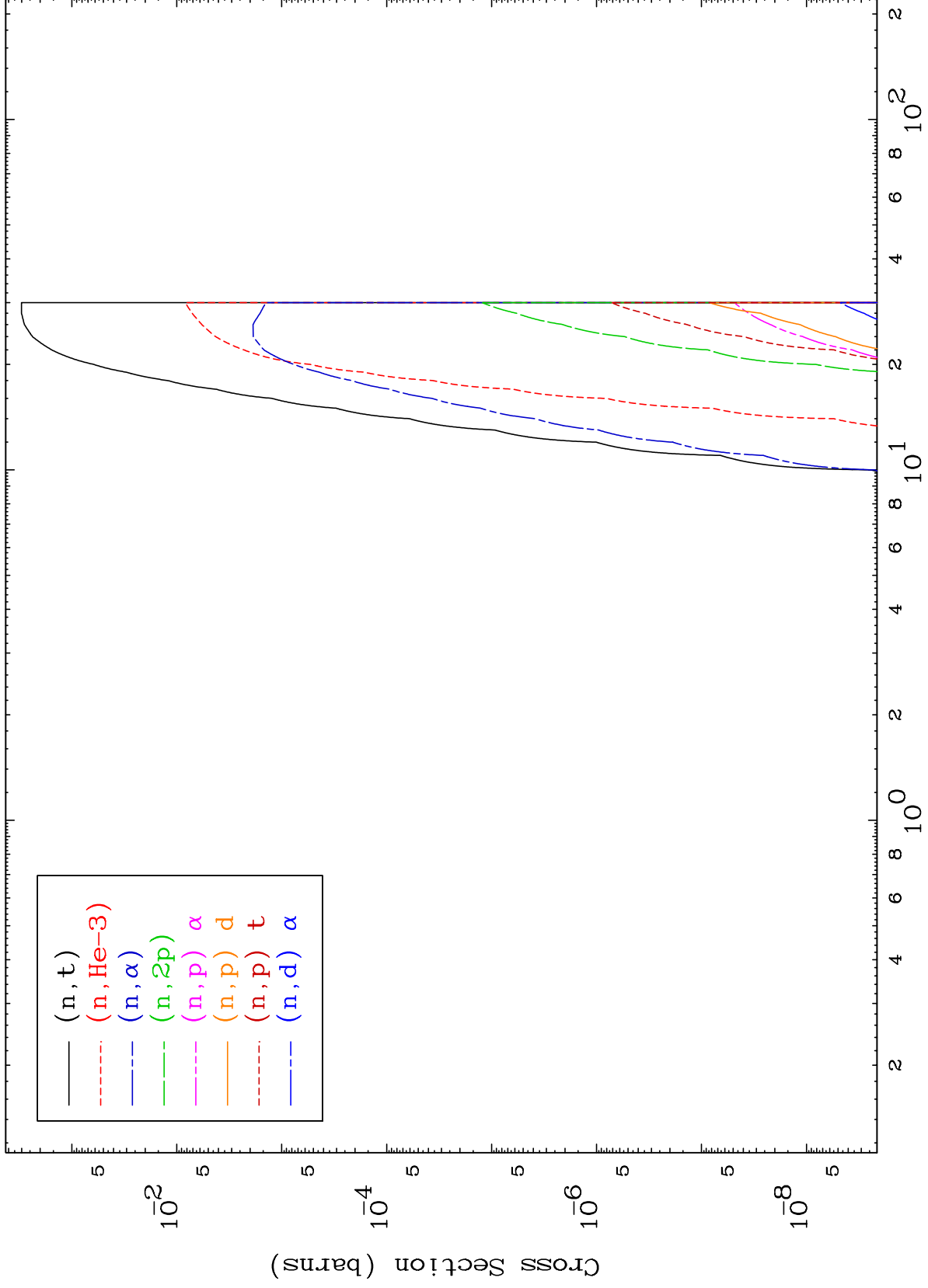
77-Ir-194



MAT 7734

He-3 Neutron Absorption  
0 Kelvin Cross Sections

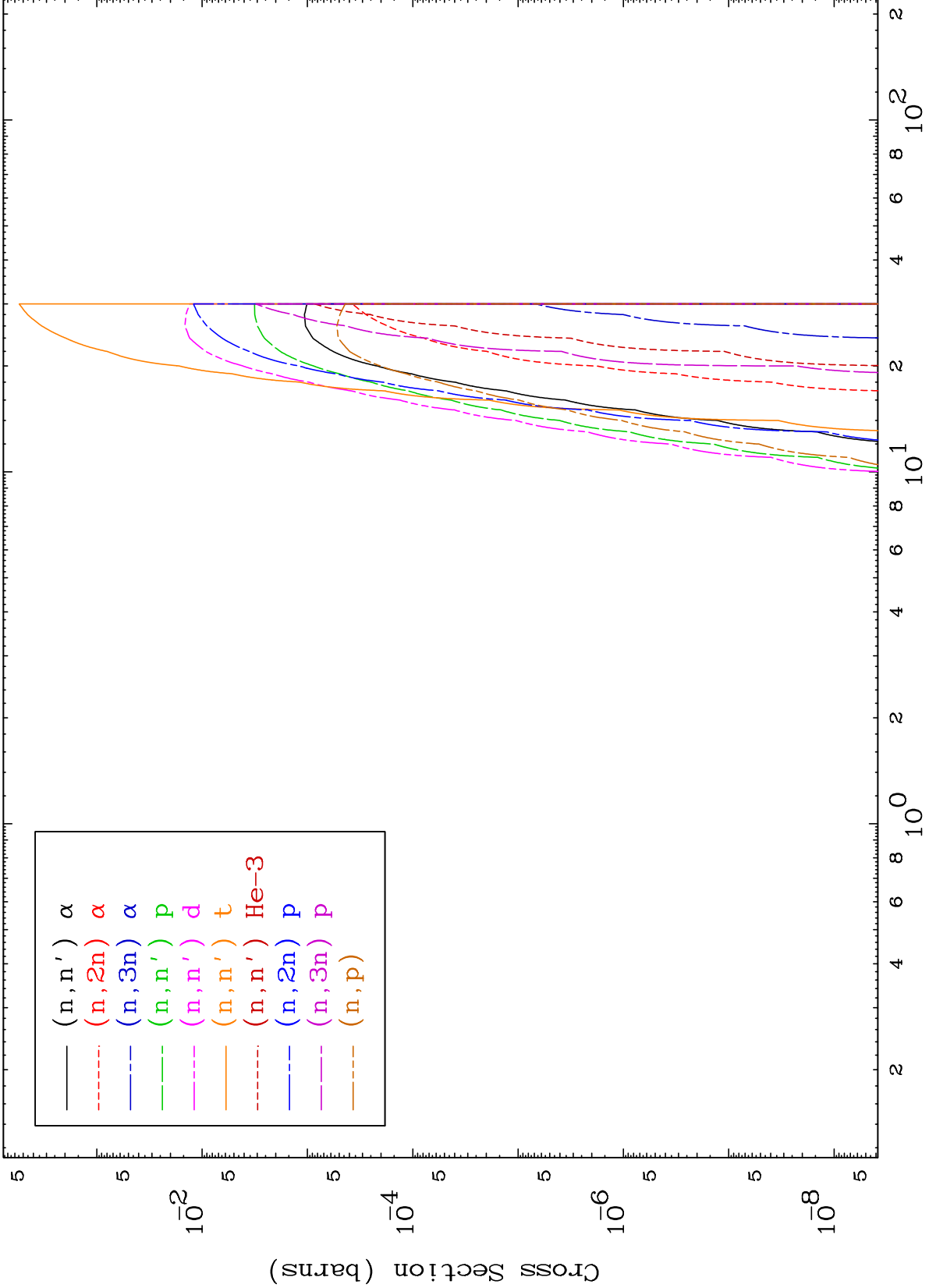
77-Ir-194



MAT 7734

He-3 Charged Particle  
0 Kelvin Cross Sections

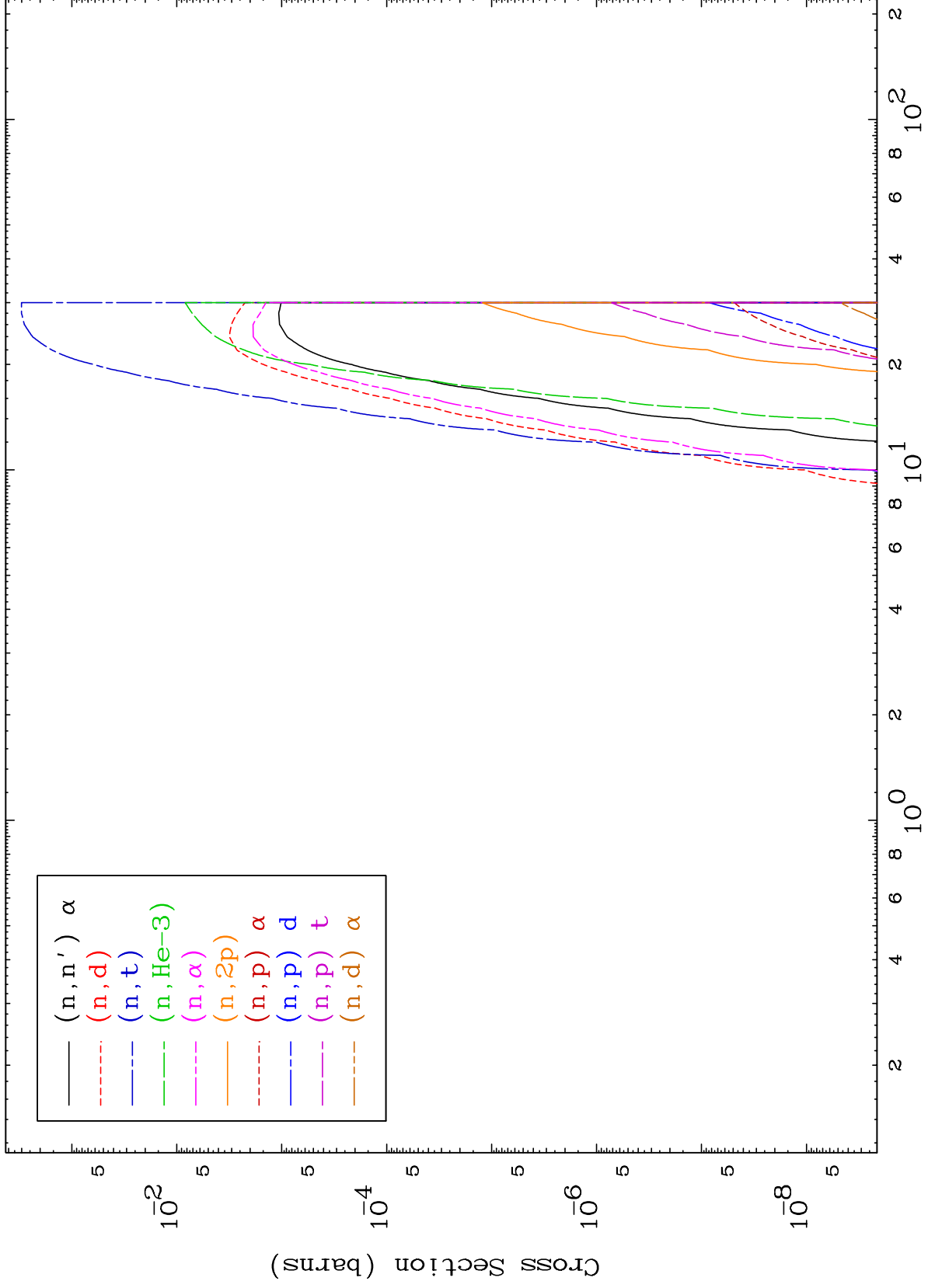
77-Ir-194



5

Incident Energy (MeV)

77-Ir-194

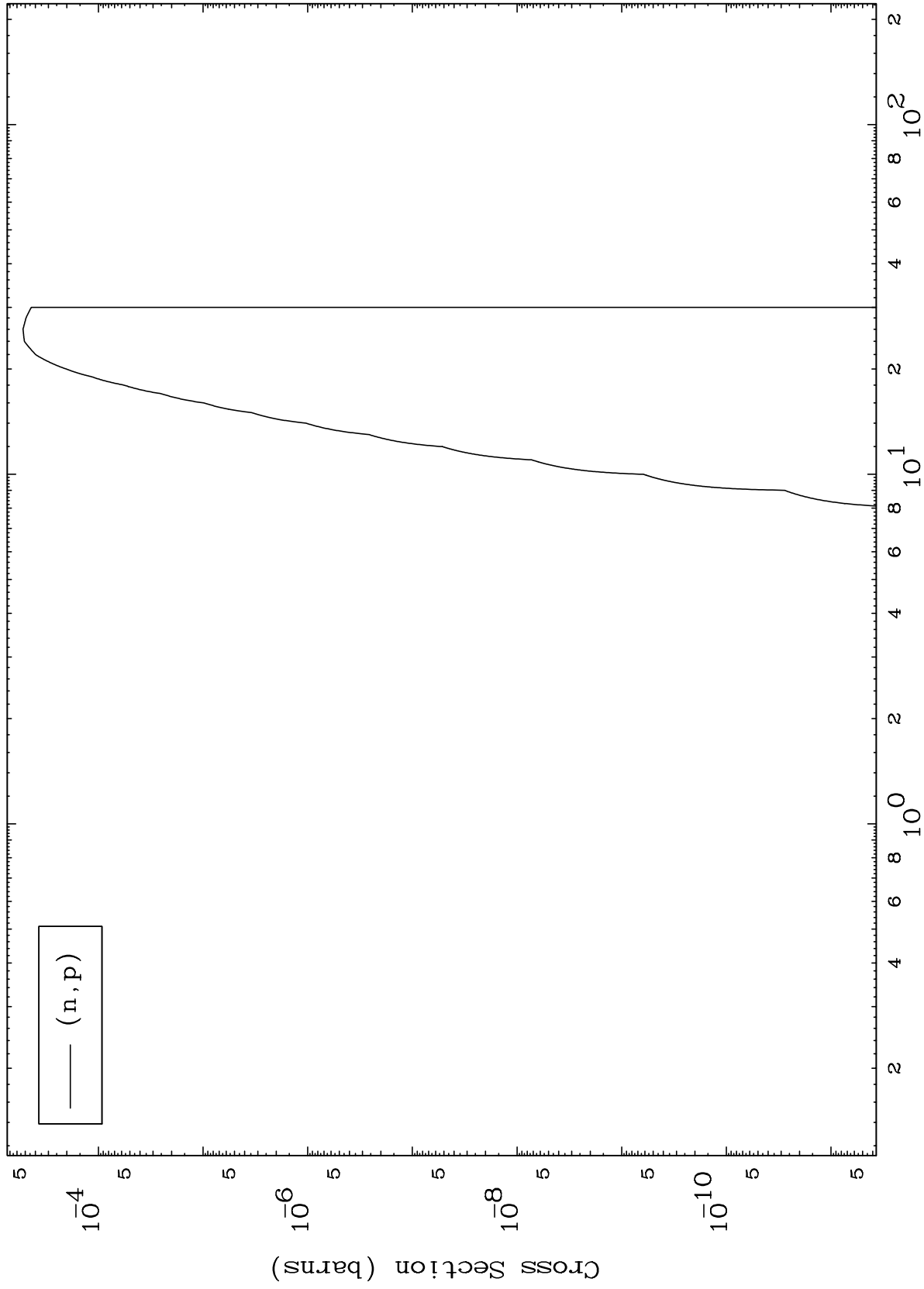


MAT 7734

(He-3,p) Levels

77-Ir-194

0 Kelvin Cross Sections

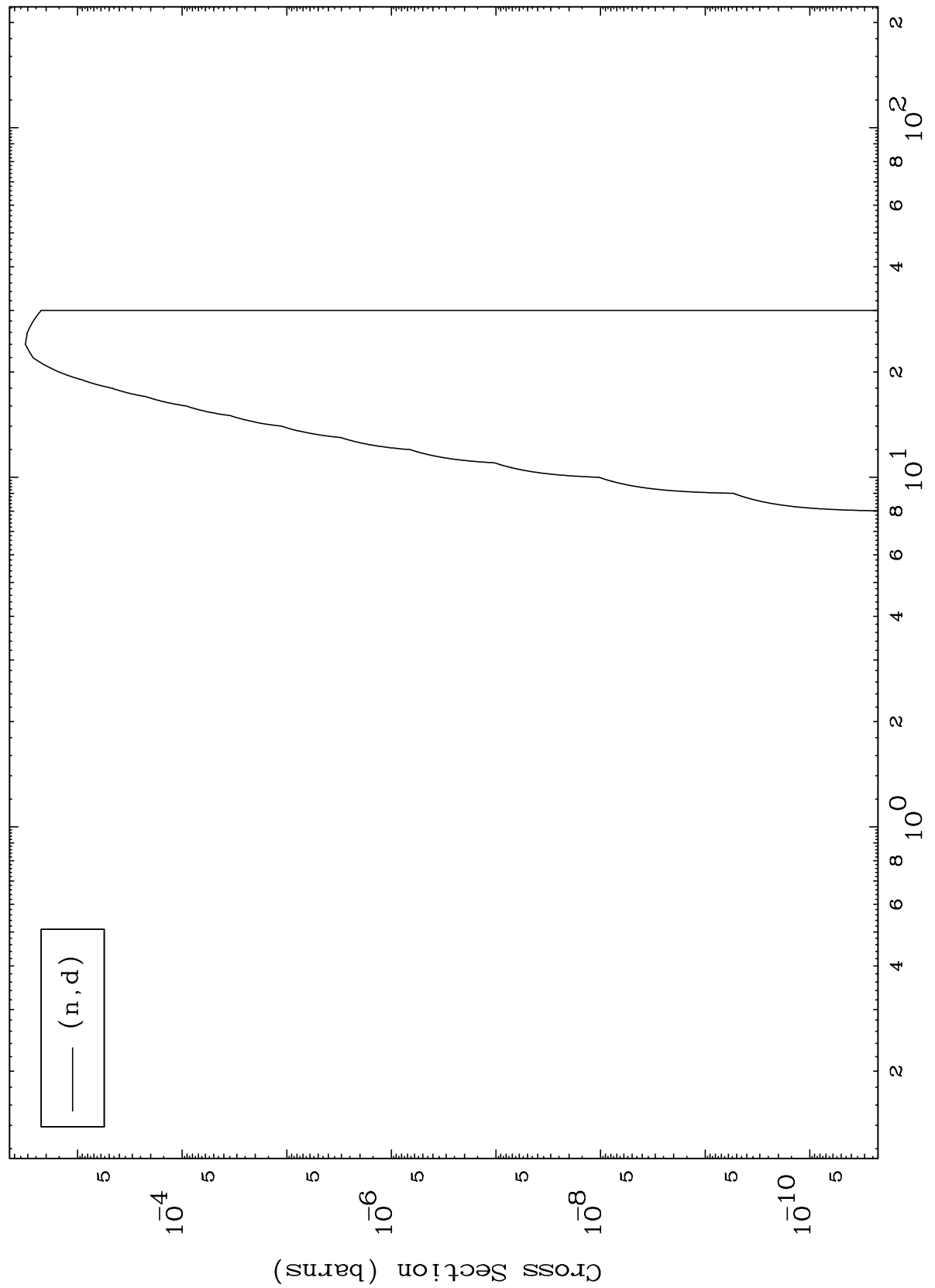


MAT 7734

(He-3,d) Levels

77-Ir-194

0 Kelvin Cross Sections

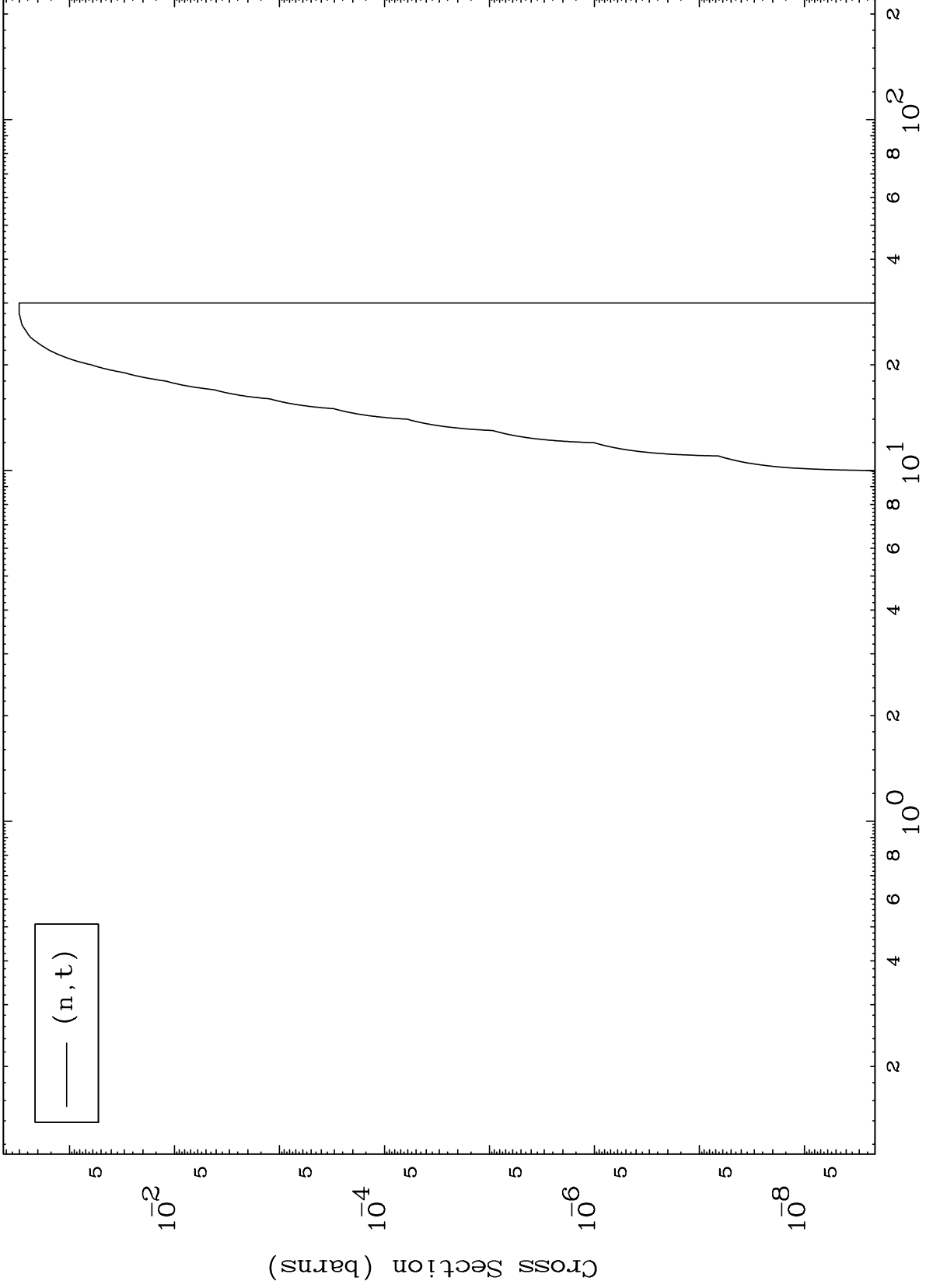


MAT 7734

(He-3,t) Levels

77-Ir-194

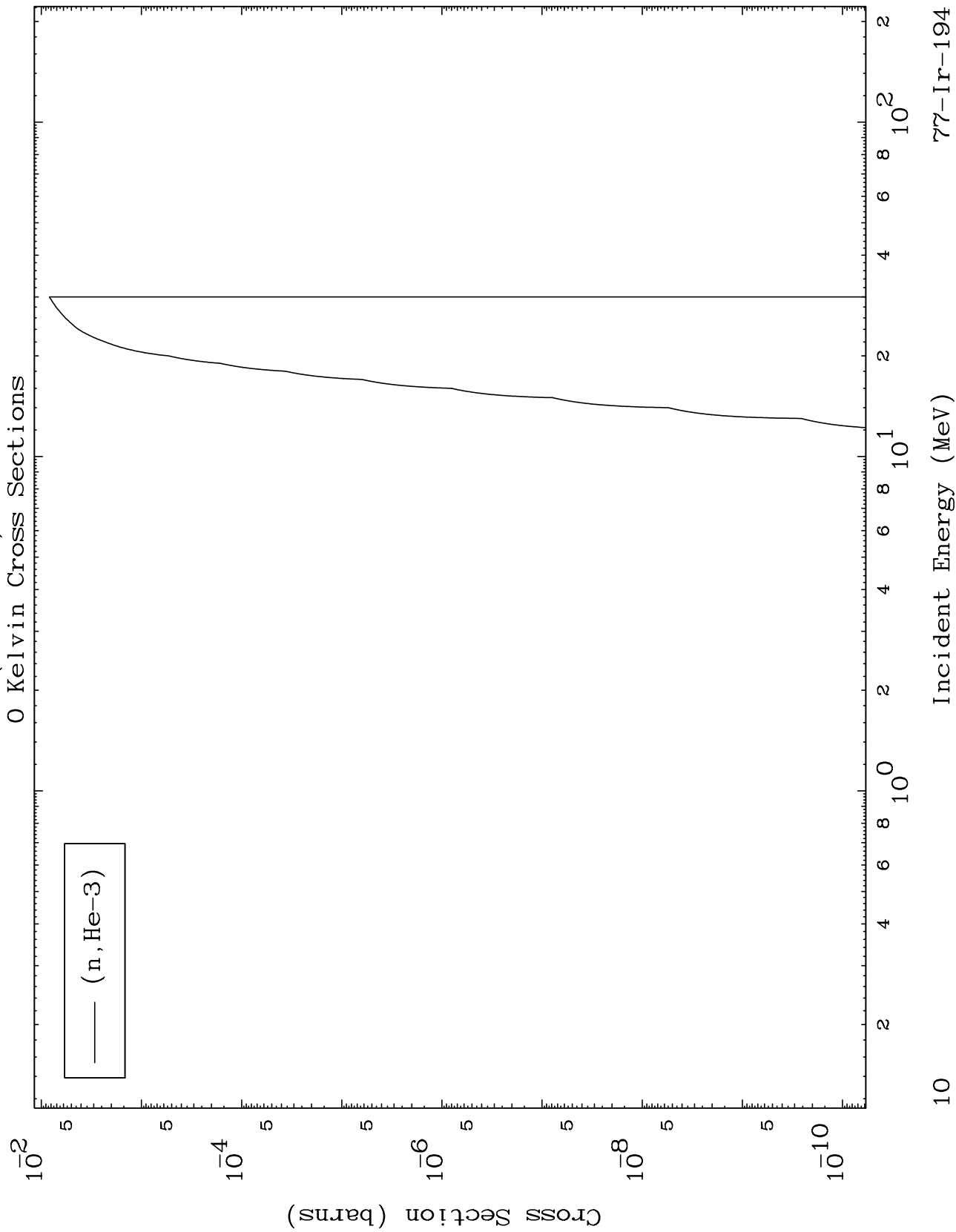
0 Kelvin Cross Sections



MAT 7734

(He-3, He3) Levels

77-Ir-194



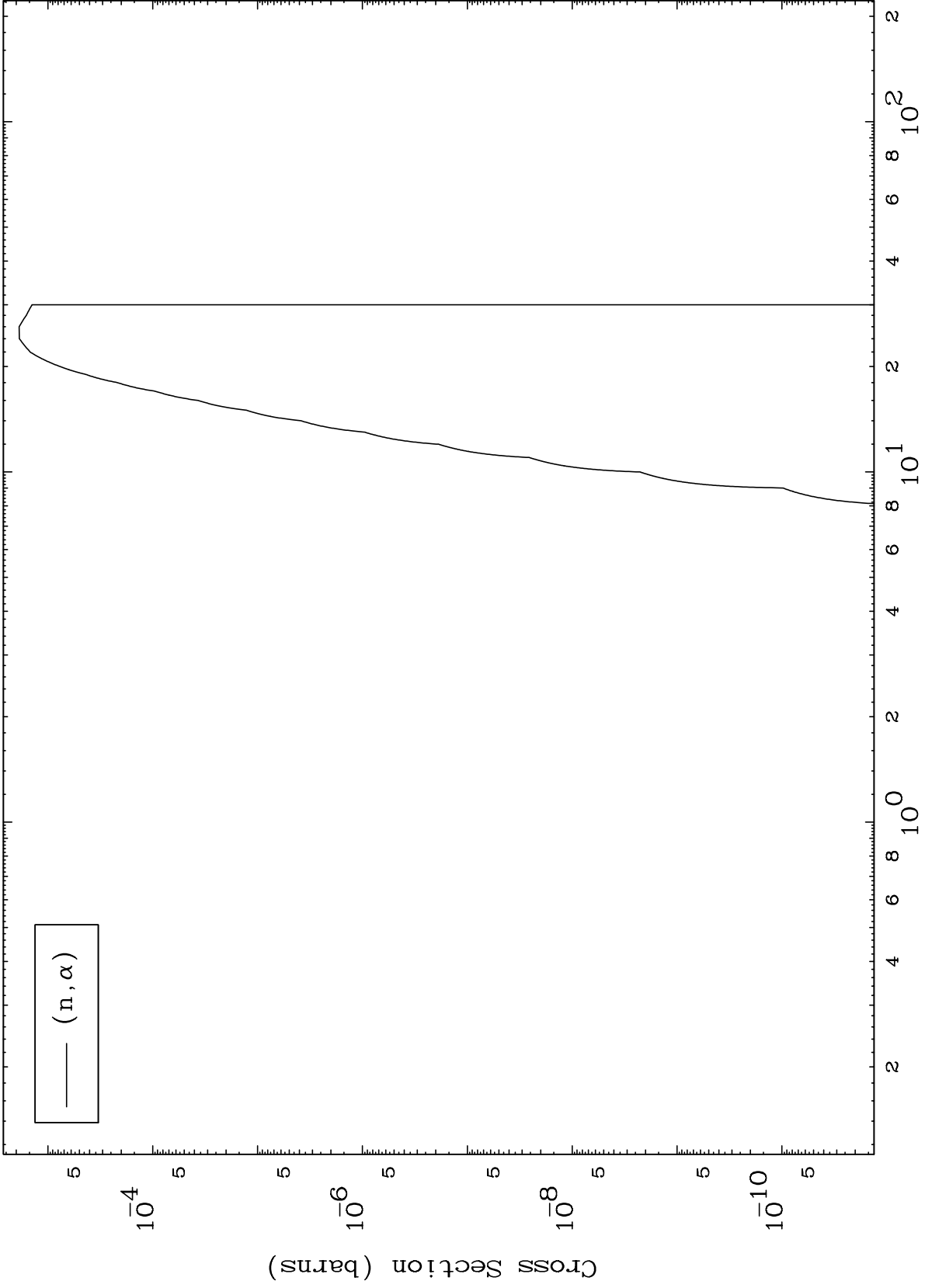
10

MAT 7734

(He-3,  $\alpha$ ) Levels

77-Ir-194

0 Kelvin Cross Sections

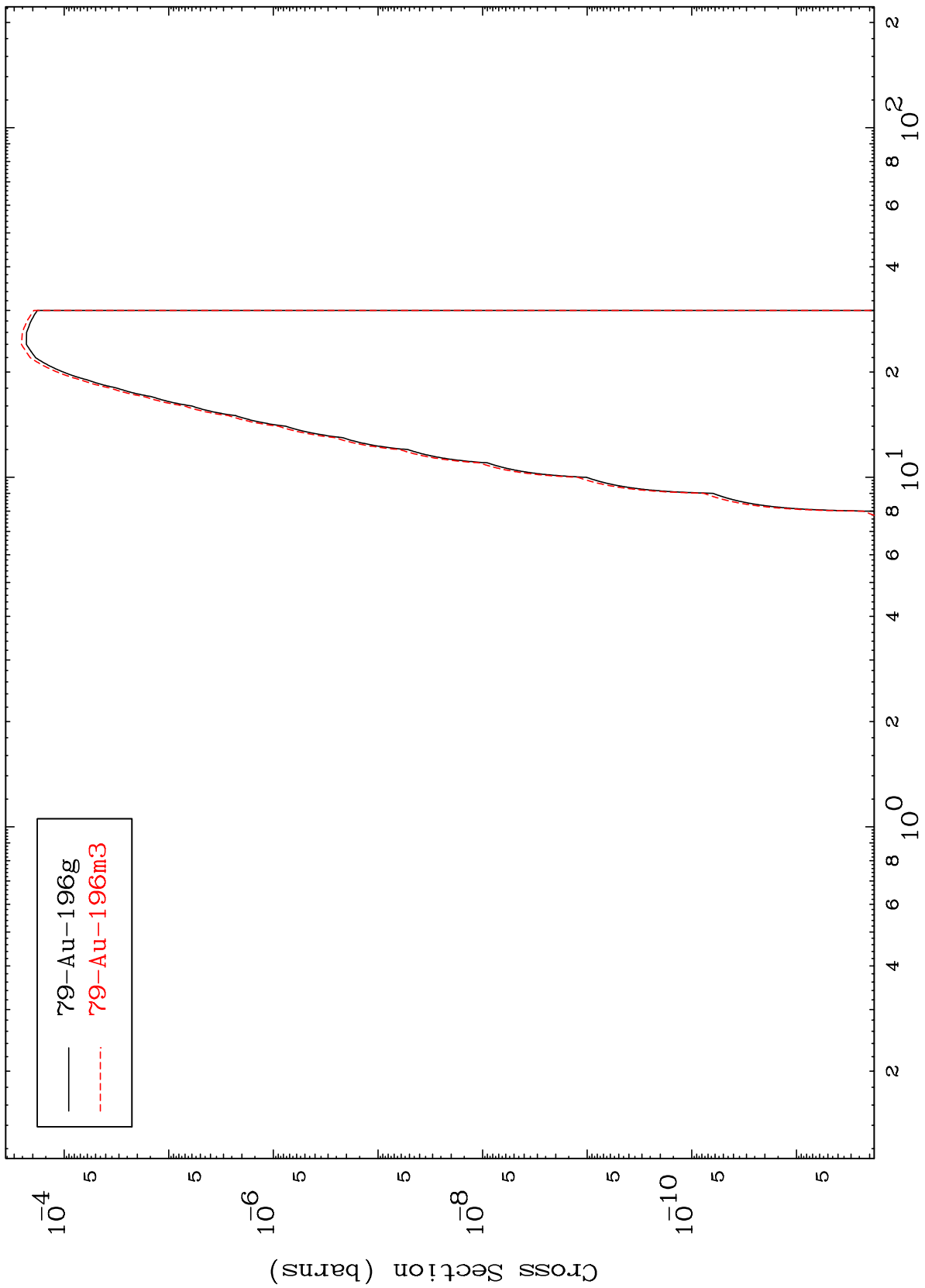


MAT 7734

Inelastic

77-Ir-194

Radionuclide Production Cross Section

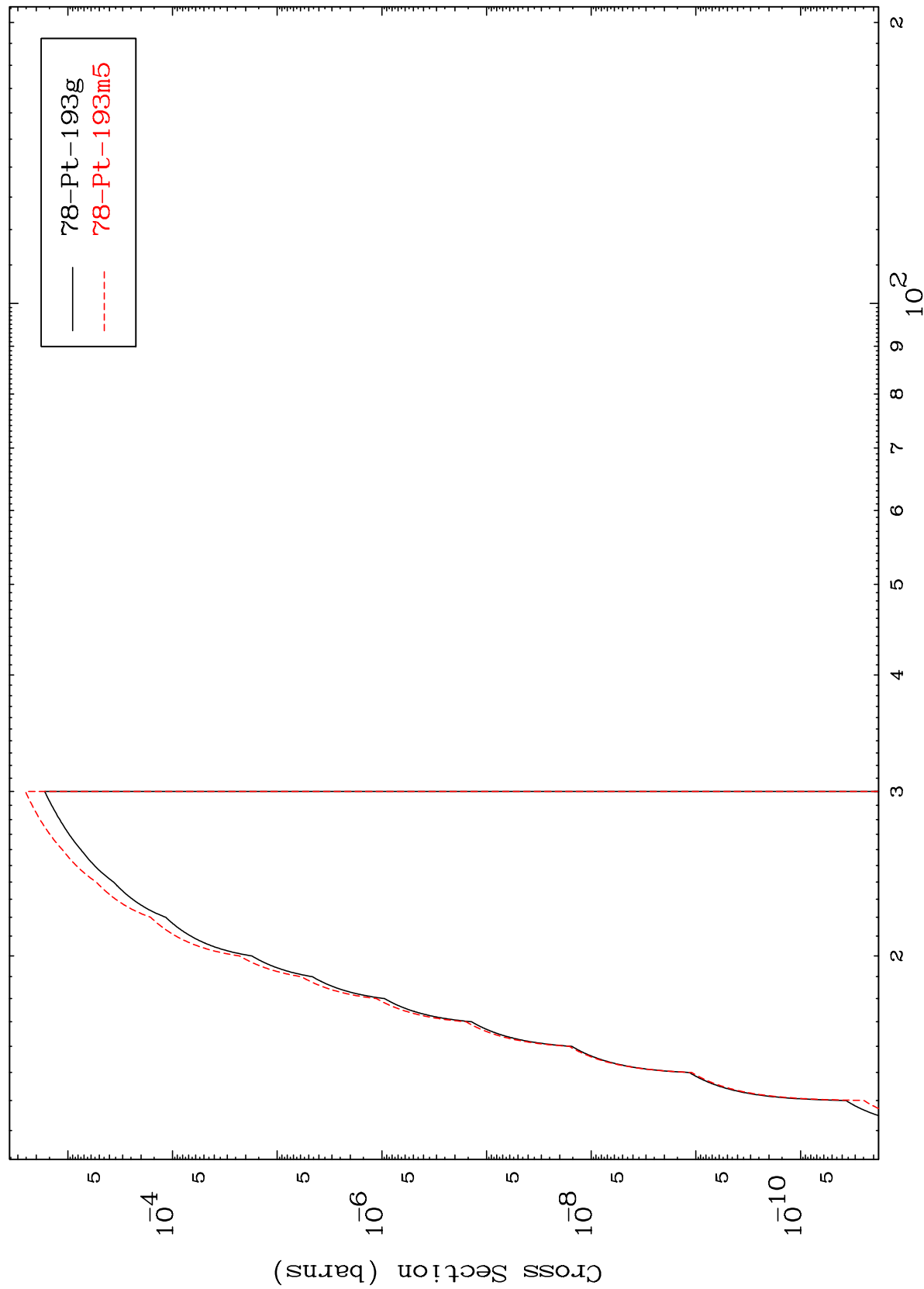


12

Incident Energy (MeV)

77-Ir-194

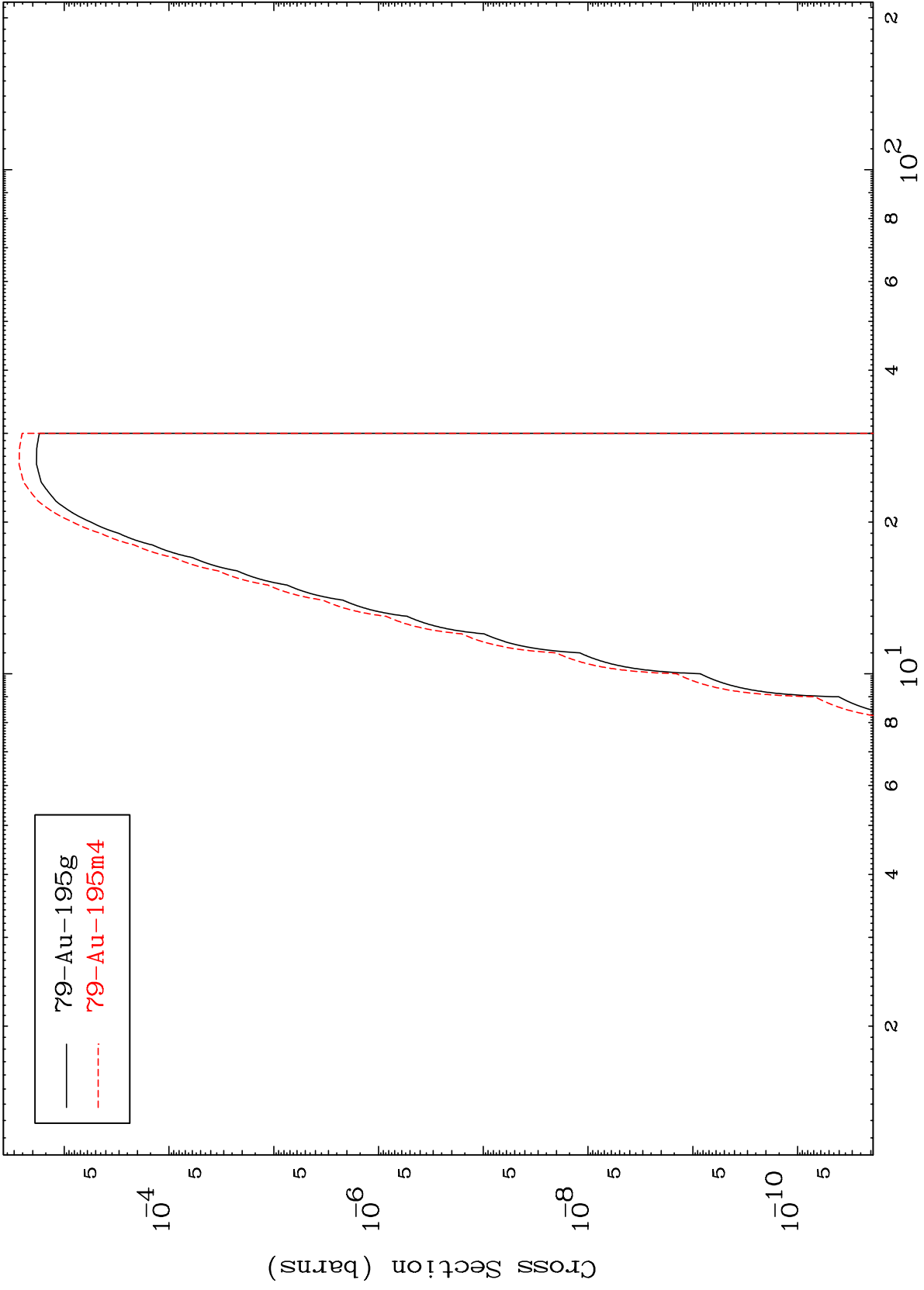
Radionuclide Production Cross Section



MAT 7734

77-Ir-194

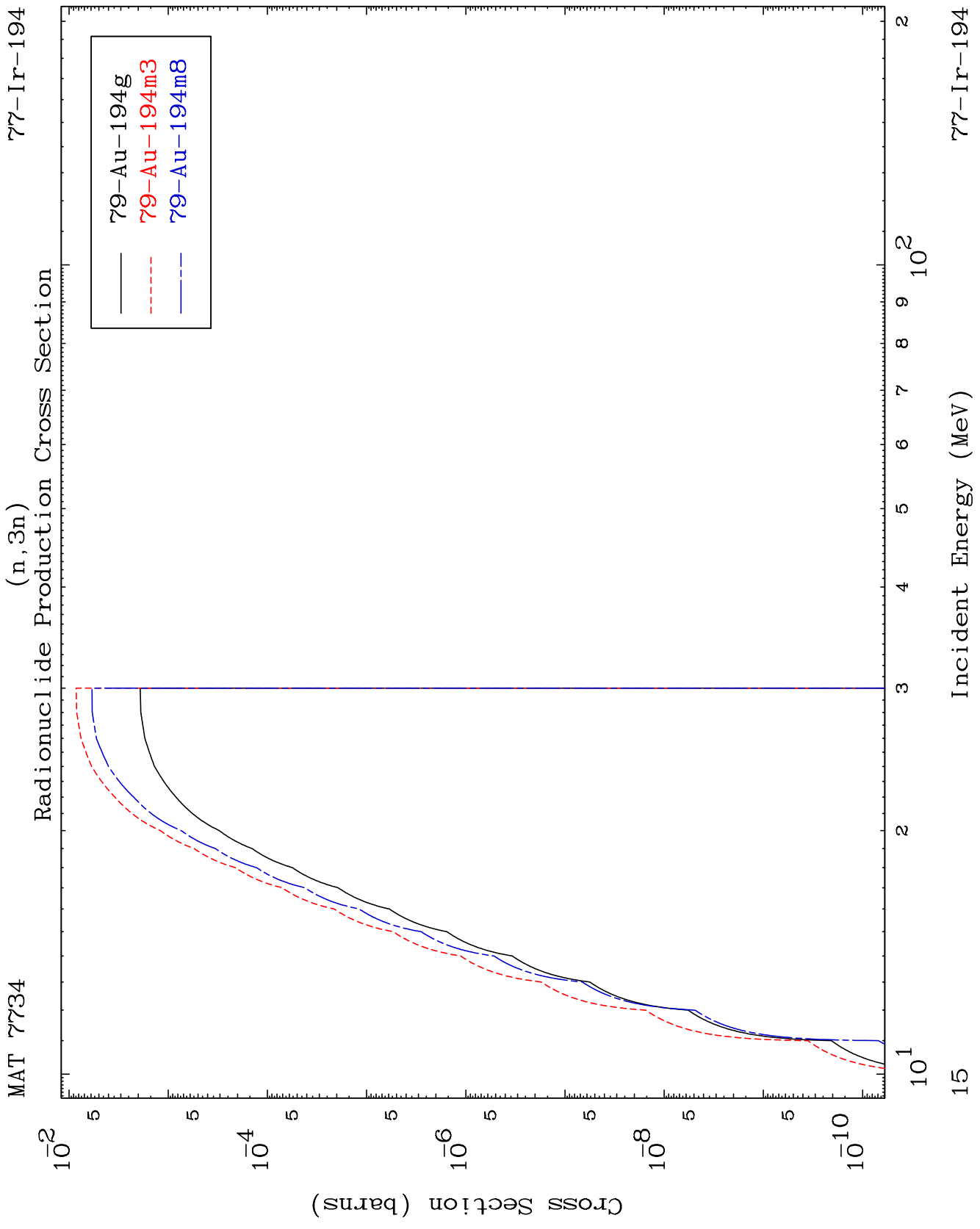
(n,2n)  
Radionuclide Production Cross Section



14

Incident Energy (MeV)

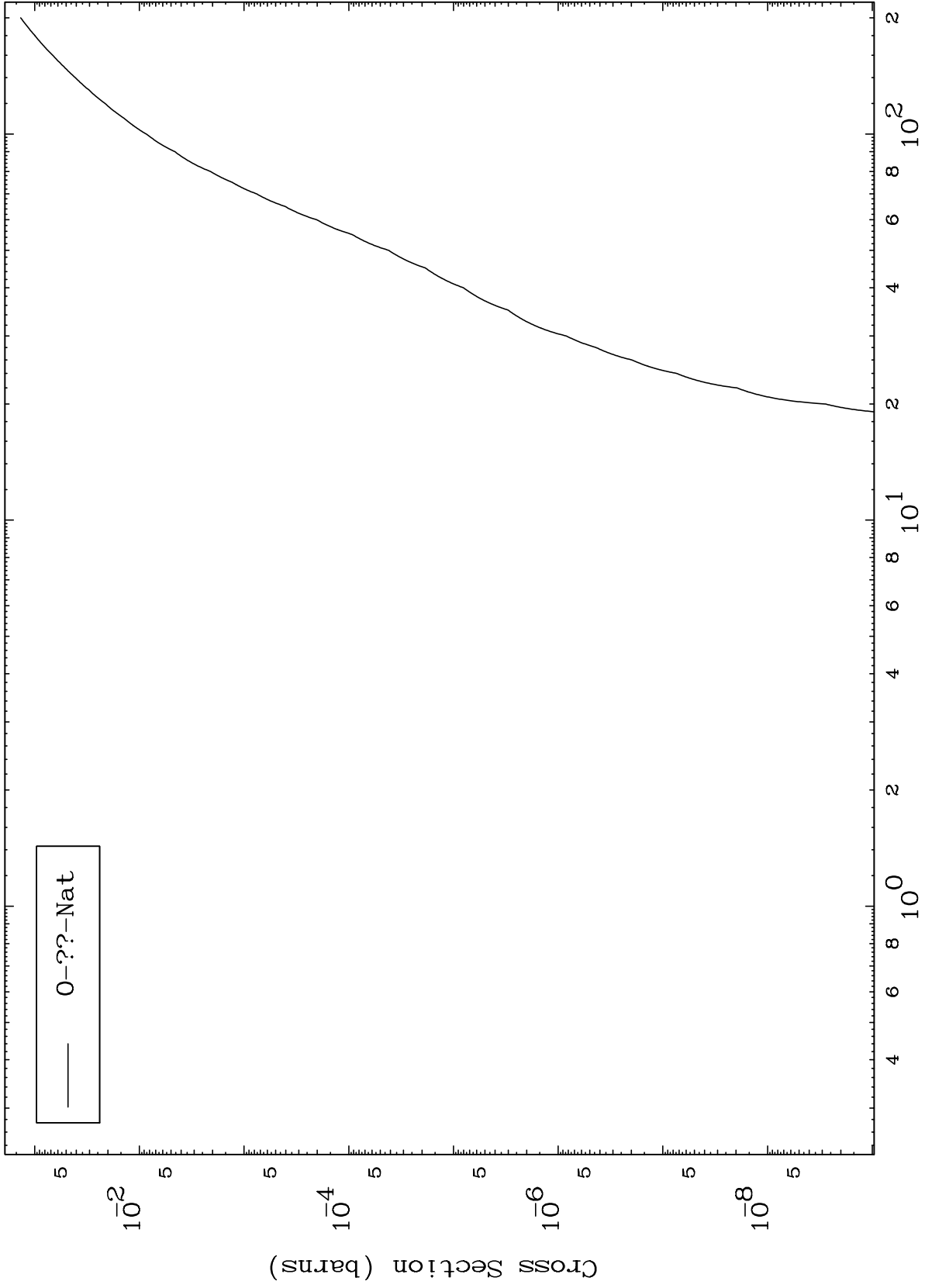
77-Ir-194



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Fission  
Radionuclide Production Cross Section

<sup>77</sup>Ir-194



16

Incident Energy (MeV)

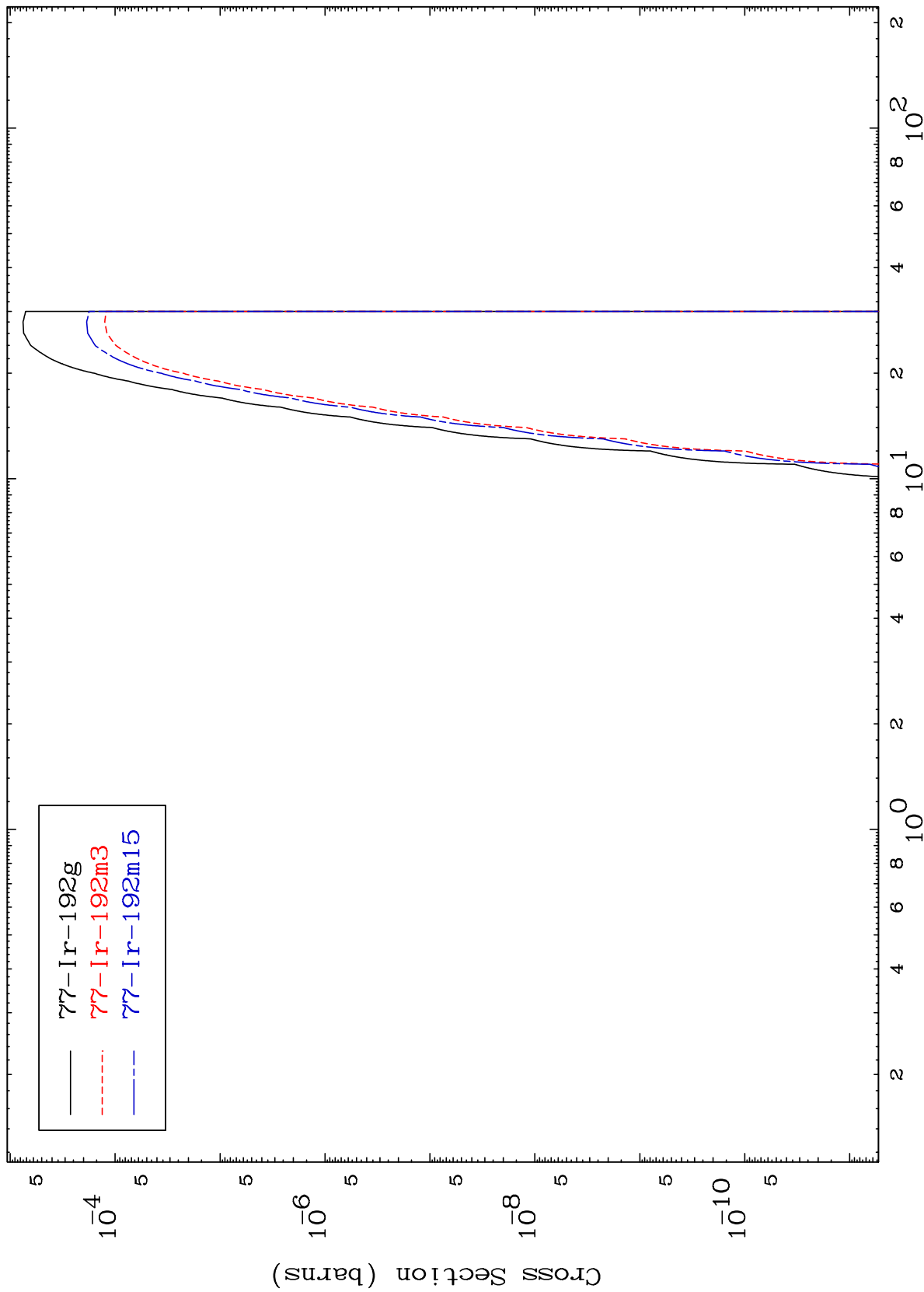
<sup>77</sup>Ir-194

MAT 7734

$(n, n') \alpha$

$^{77}\text{Ir-194}$

Radionuclide Production Cross Section

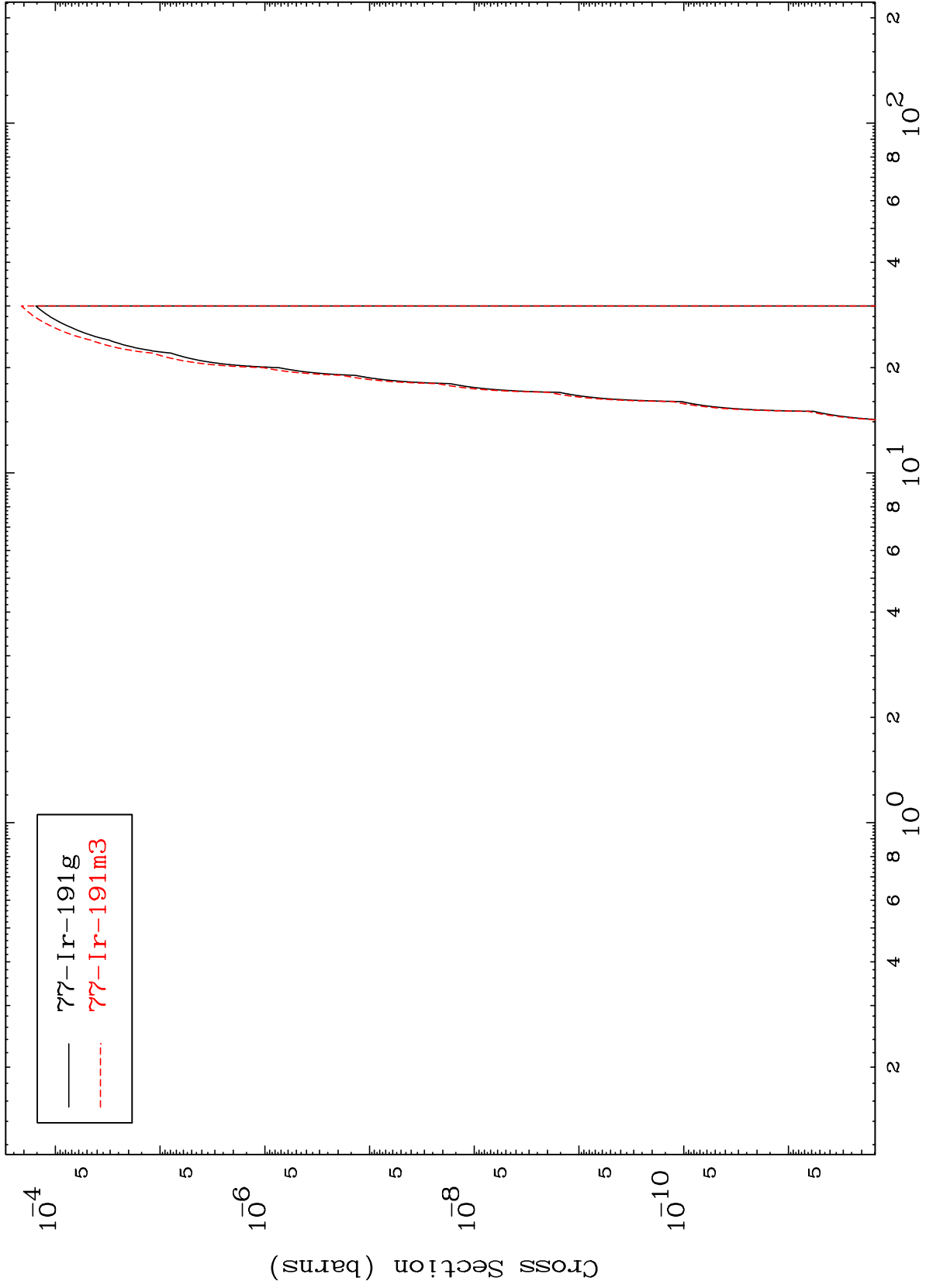


MAT 7734

(n,2n)  $\alpha$

<sup>77</sup>Ir-194

Radionuclide Production Cross Section

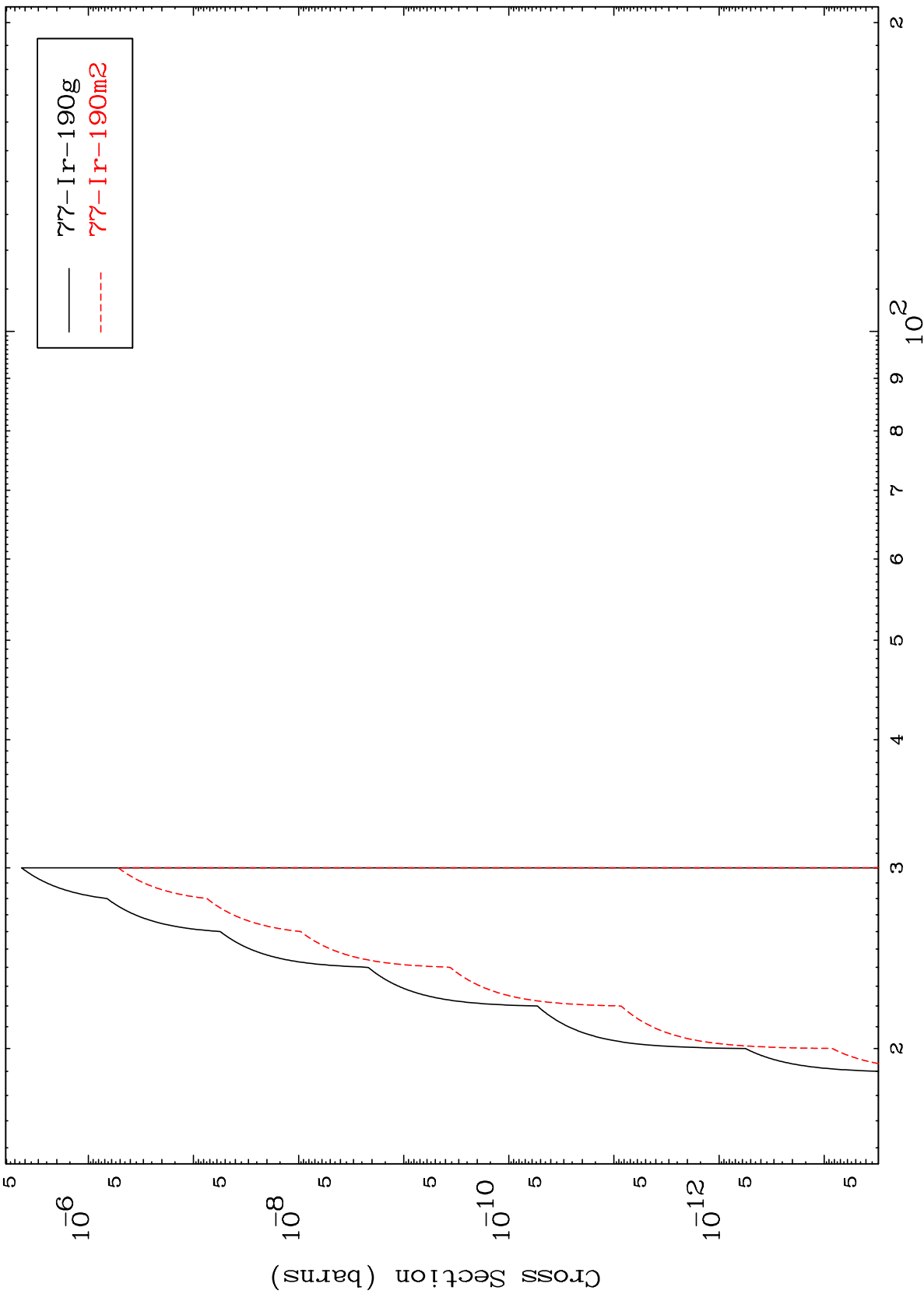


MAT 7734

$(n,3n) \alpha$

$^{77}\text{Ir-194}$

Radionuclide Production Cross Section



19

Incident Energy (MeV)

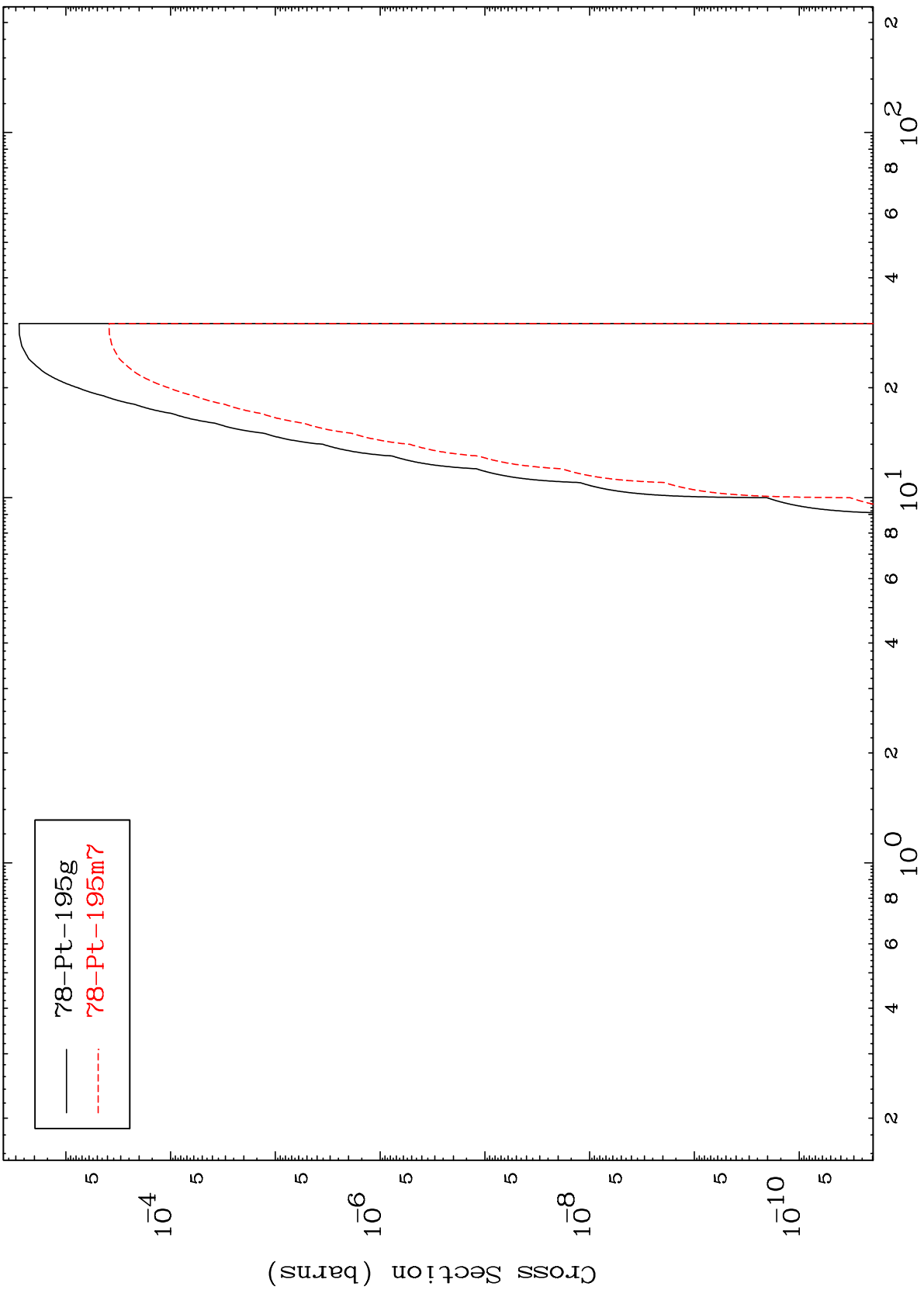
$^{77}\text{Ir-194}$

MAT 7734

$(n, n')$  p

$^{77}\text{Ir-194}$

Radionuclide Production Cross Section



20

Incident Energy (MeV)

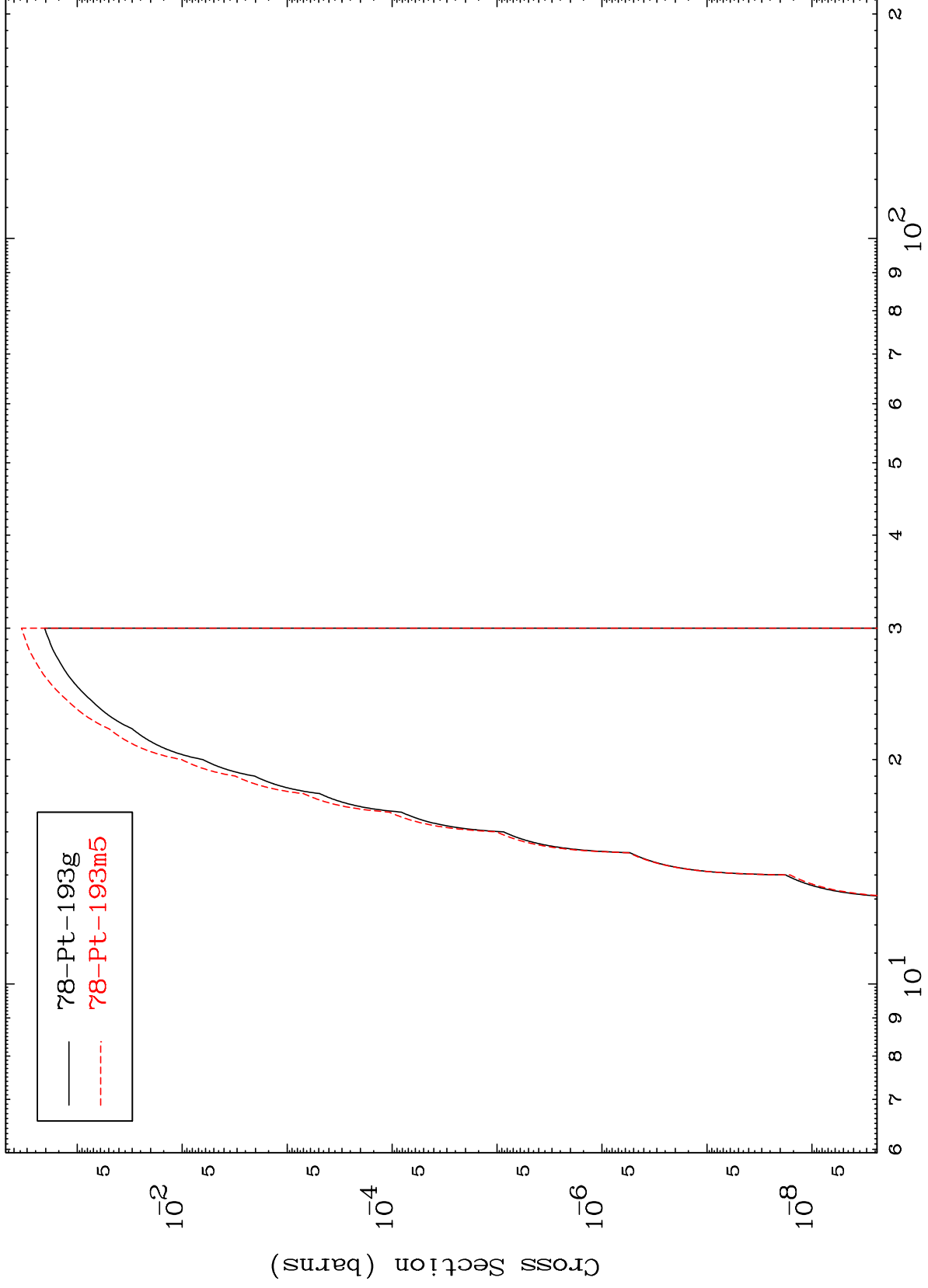
$^{77}\text{Ir-194}$

MAT 7734

(n,n') t

<sup>77</sup>Ir-194

Radionuclide Production Cross Section



21

Incident Energy (MeV)

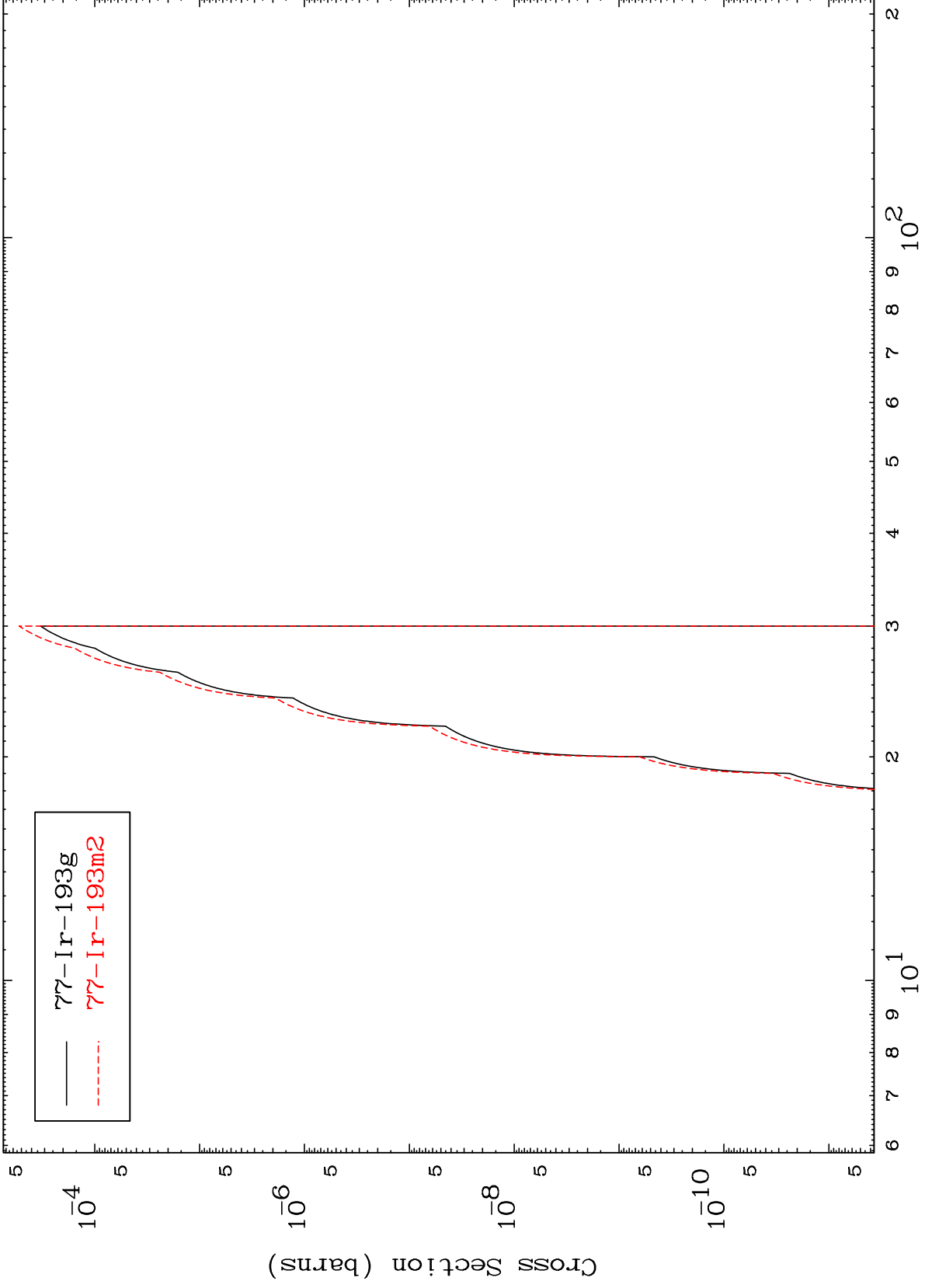
<sup>77</sup>Ir-194

MAT 7734

(n,n') He-3

77-Ir-194

Radionuclide Production Cross Section



22

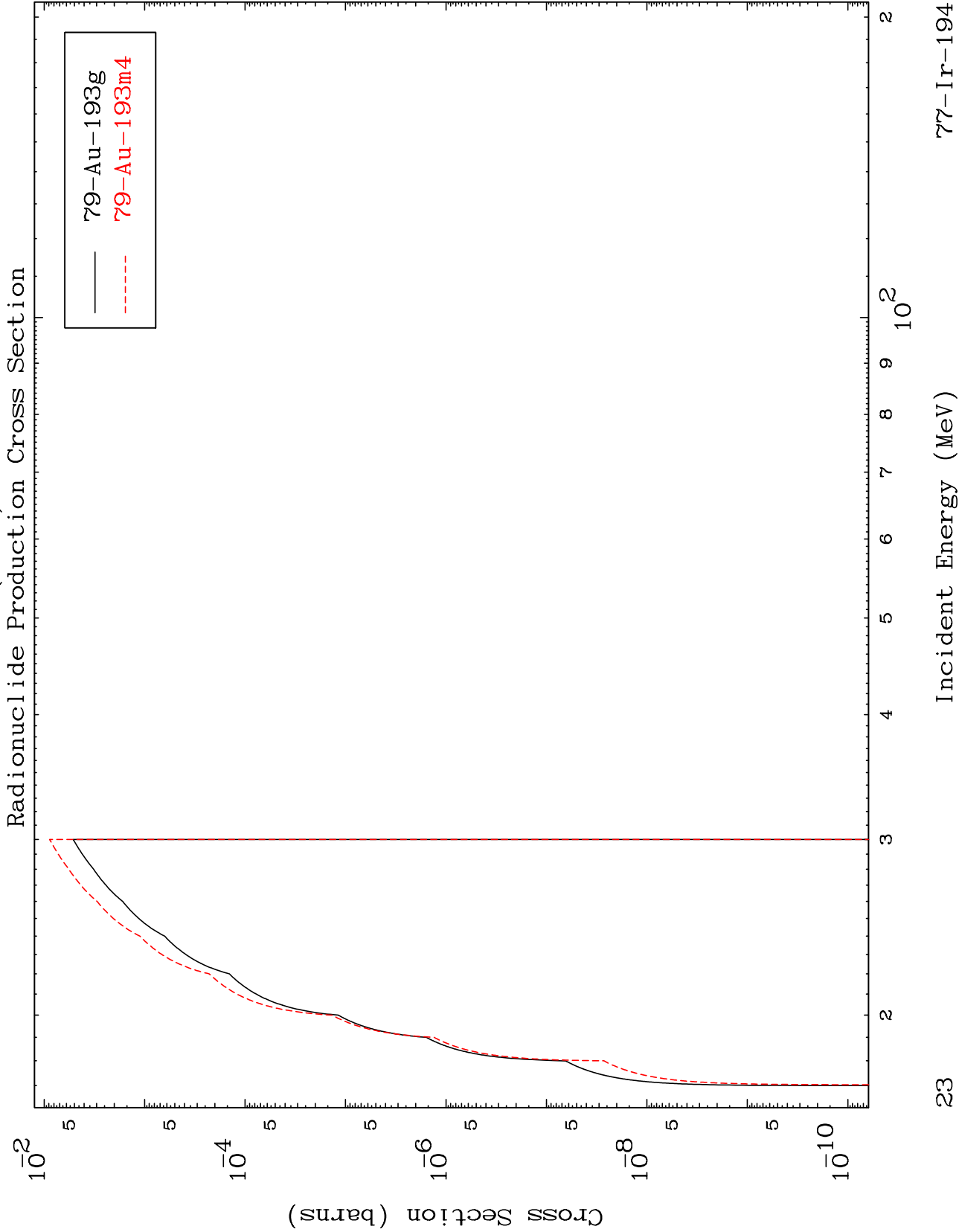
Incident Energy (MeV)

77-Ir-194

MAT 7734

(n,4n)

77-Ir-194



23

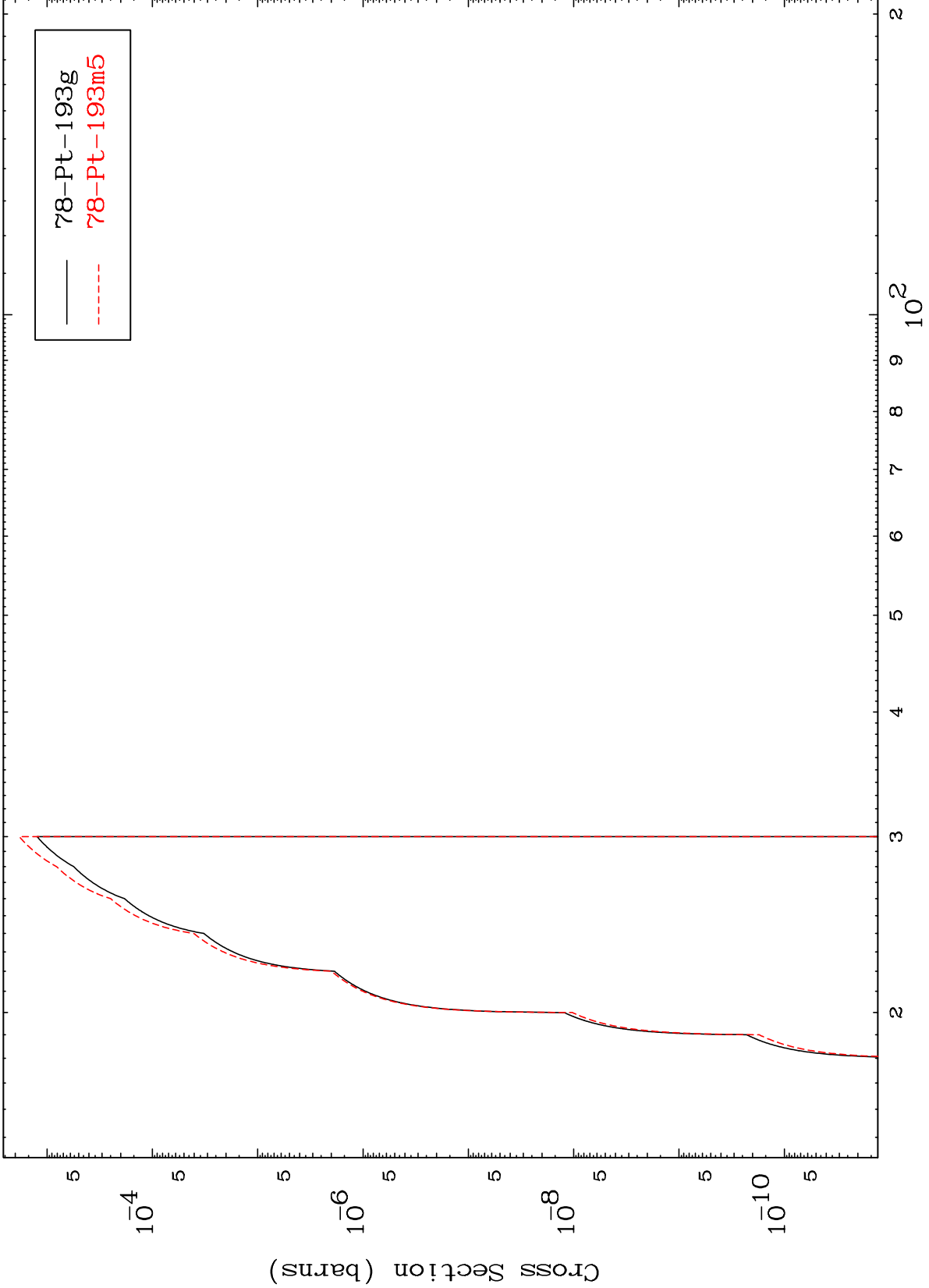
77-Ir-194

MAT 7734

(n,3n) p

77-Ir-194

Radionuclide Production Cross Section



78-Pt-193g  
78-Pt-193m5

24

Incident Energy (MeV)

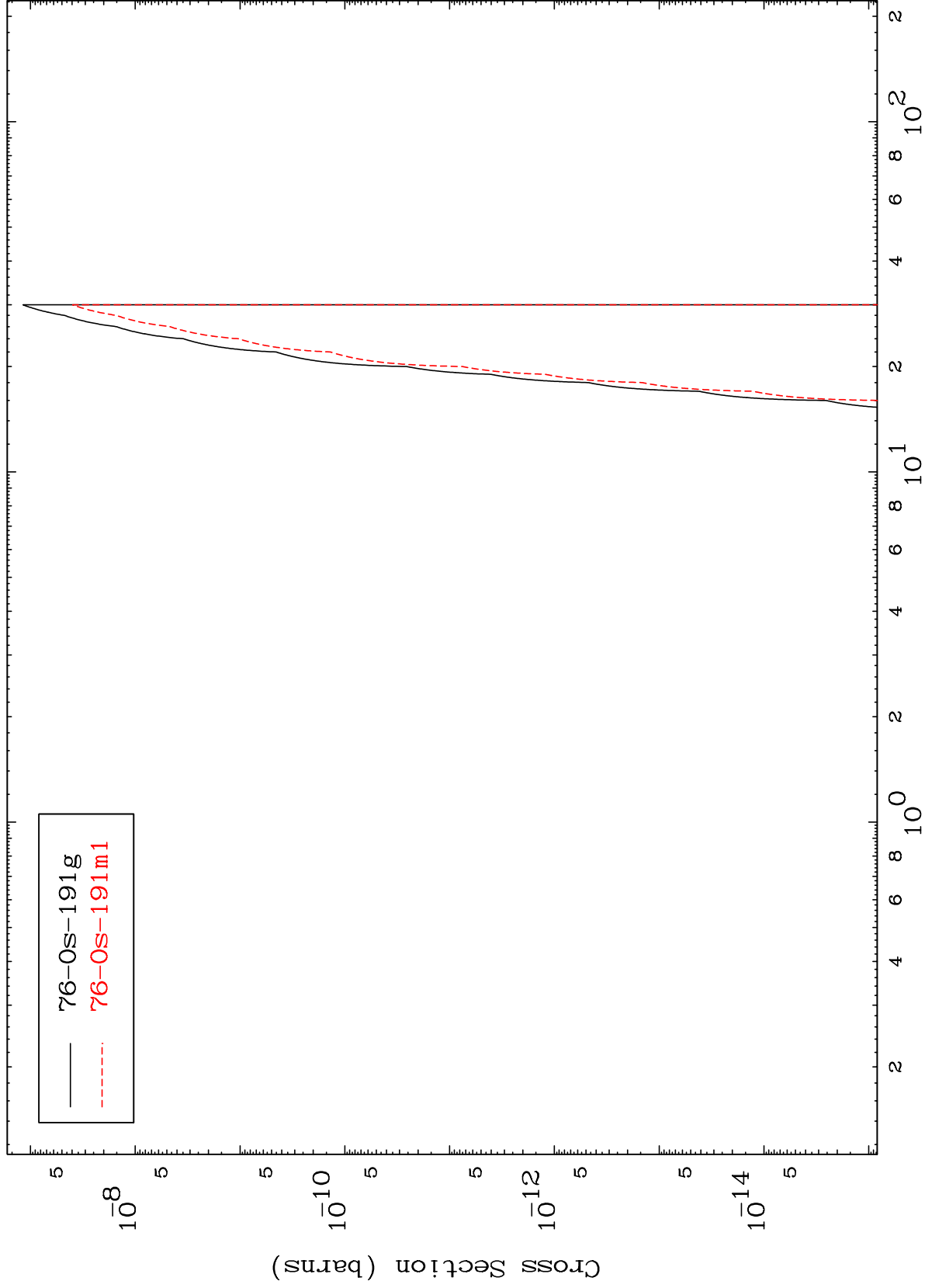
77-Ir-194

MAT 7734

(n,n') p  $\alpha$

77-Ir-194

Radionuclide Production Cross Section



25

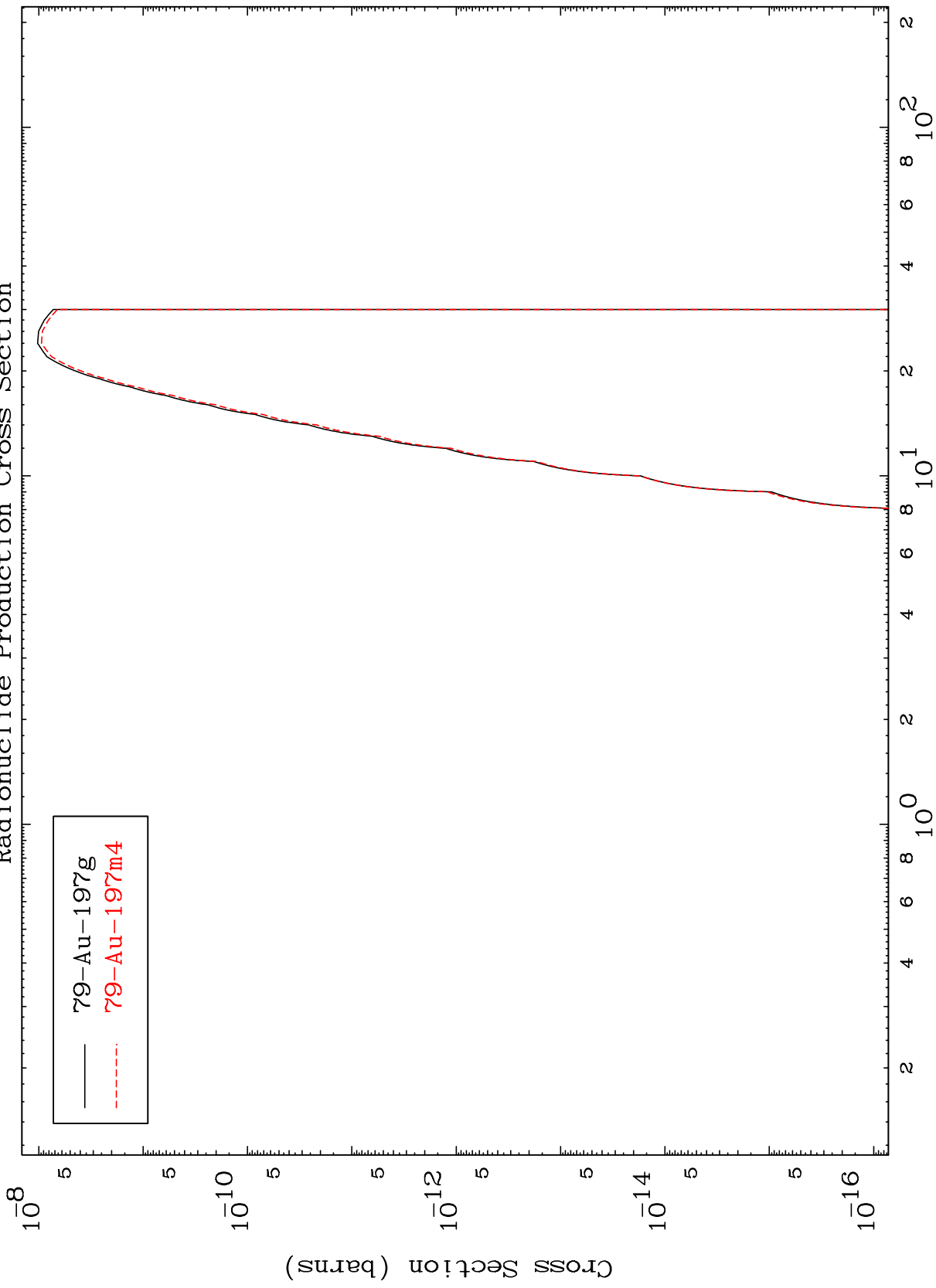
Incident Energy (MeV)

77-Ir-194

MAT 7734

77-Ir-194

Radionuclide Production Cross Section



Incident Energy (MeV)

77-Ir-194

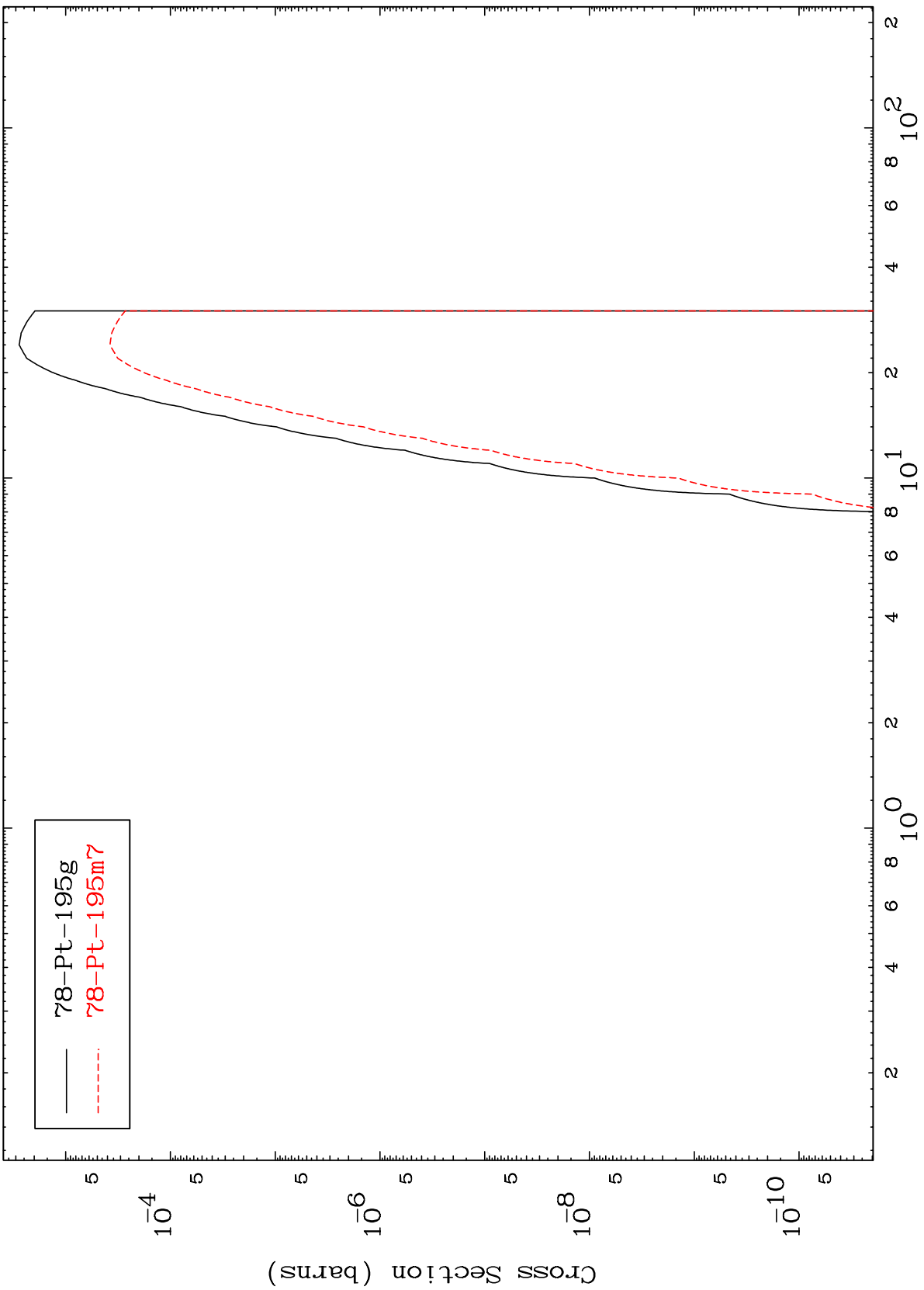
26

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(n,d)

<sup>77</sup>Ir-194

Radionuclide Production Cross Section

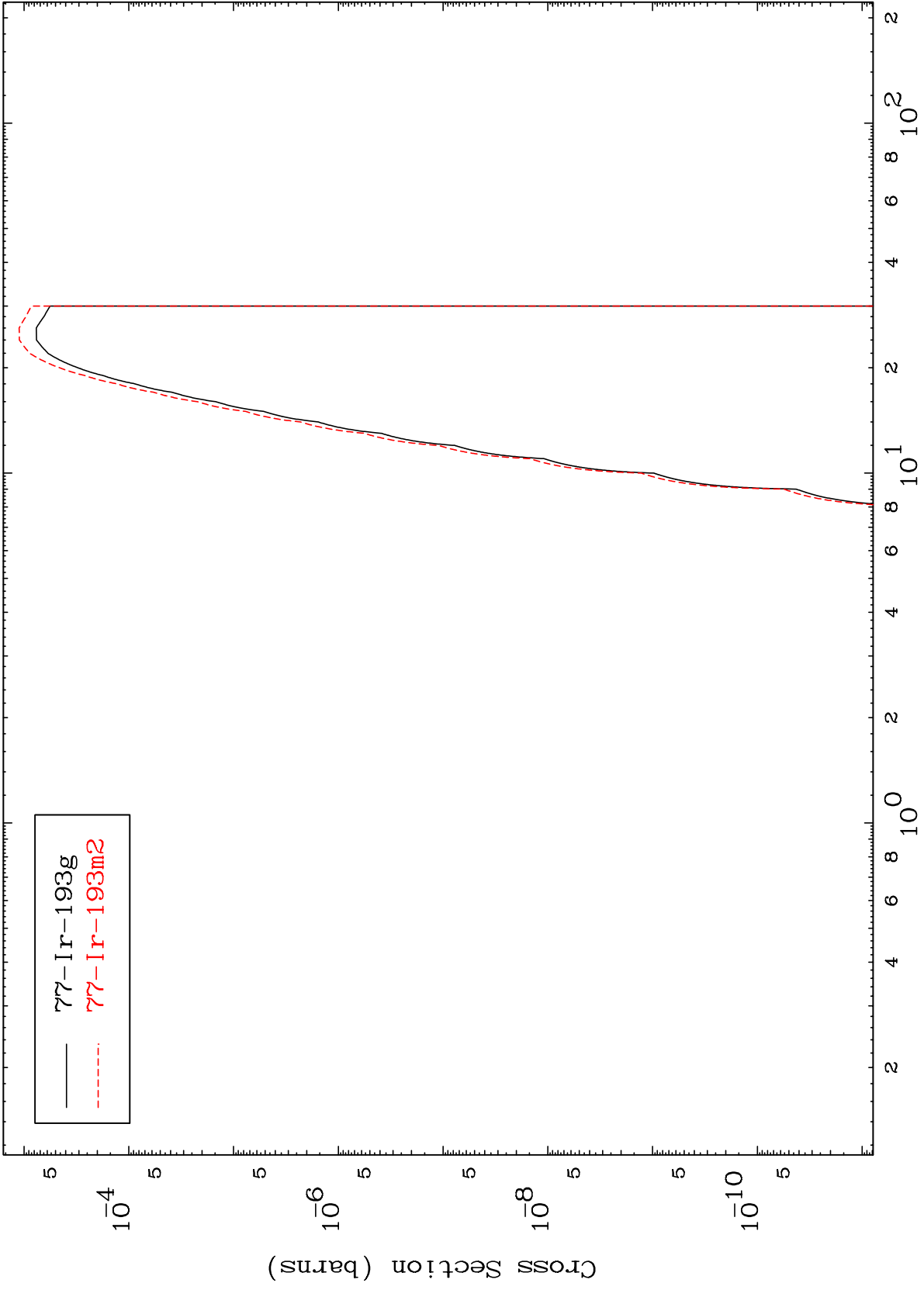


— 78-Pt-195g  
- - - 78-Pt-195m7

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<sup>77</sup>Ir-194

(n,α)  
Radionuclide Production Cross Section



<sup>77</sup>Ir-194

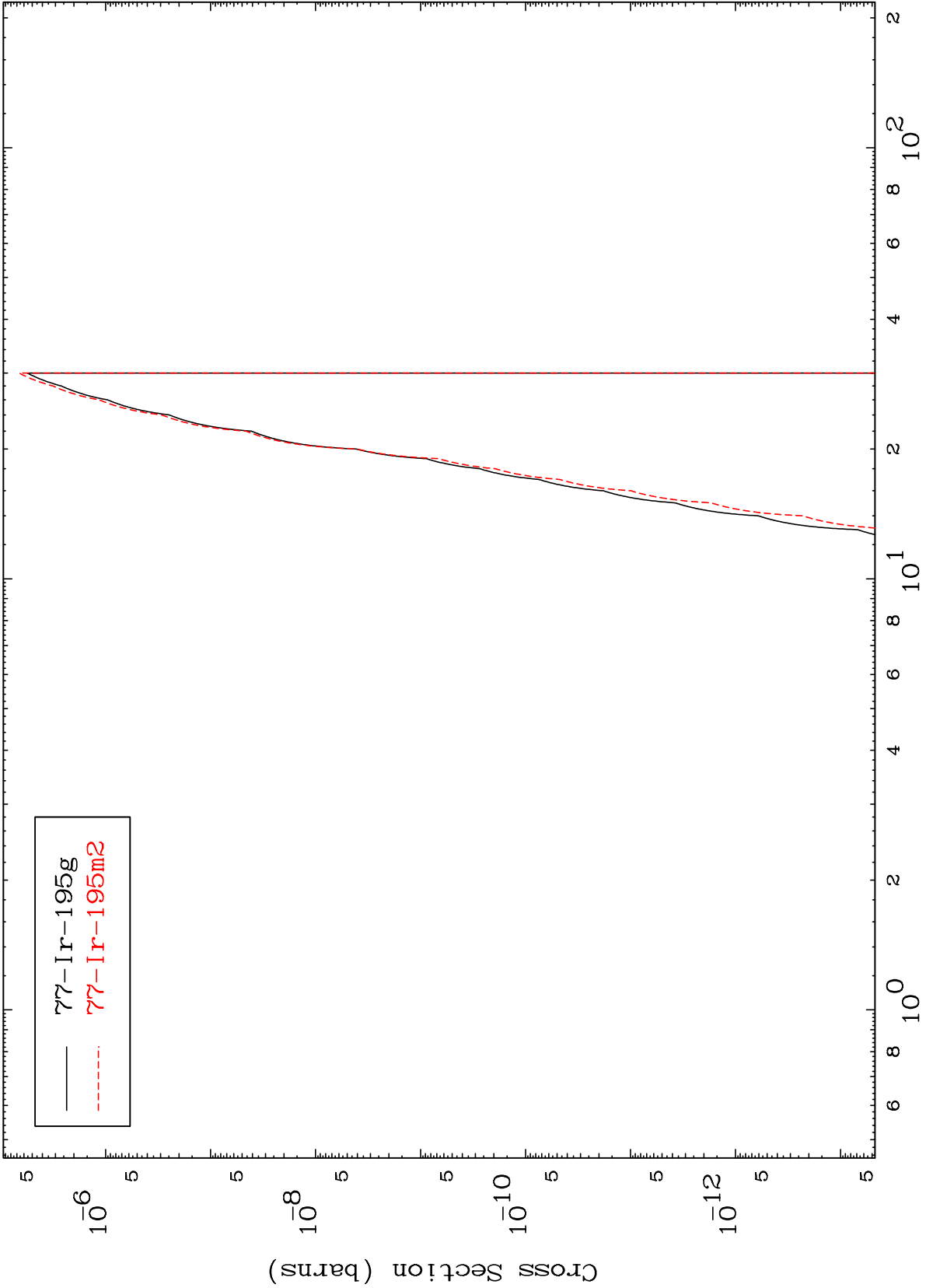
Incident Energy (MeV)

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(n,2p)

<sup>77</sup>Ir-194

Radionuclide Production Cross Section



29

Incident Energy (MeV)

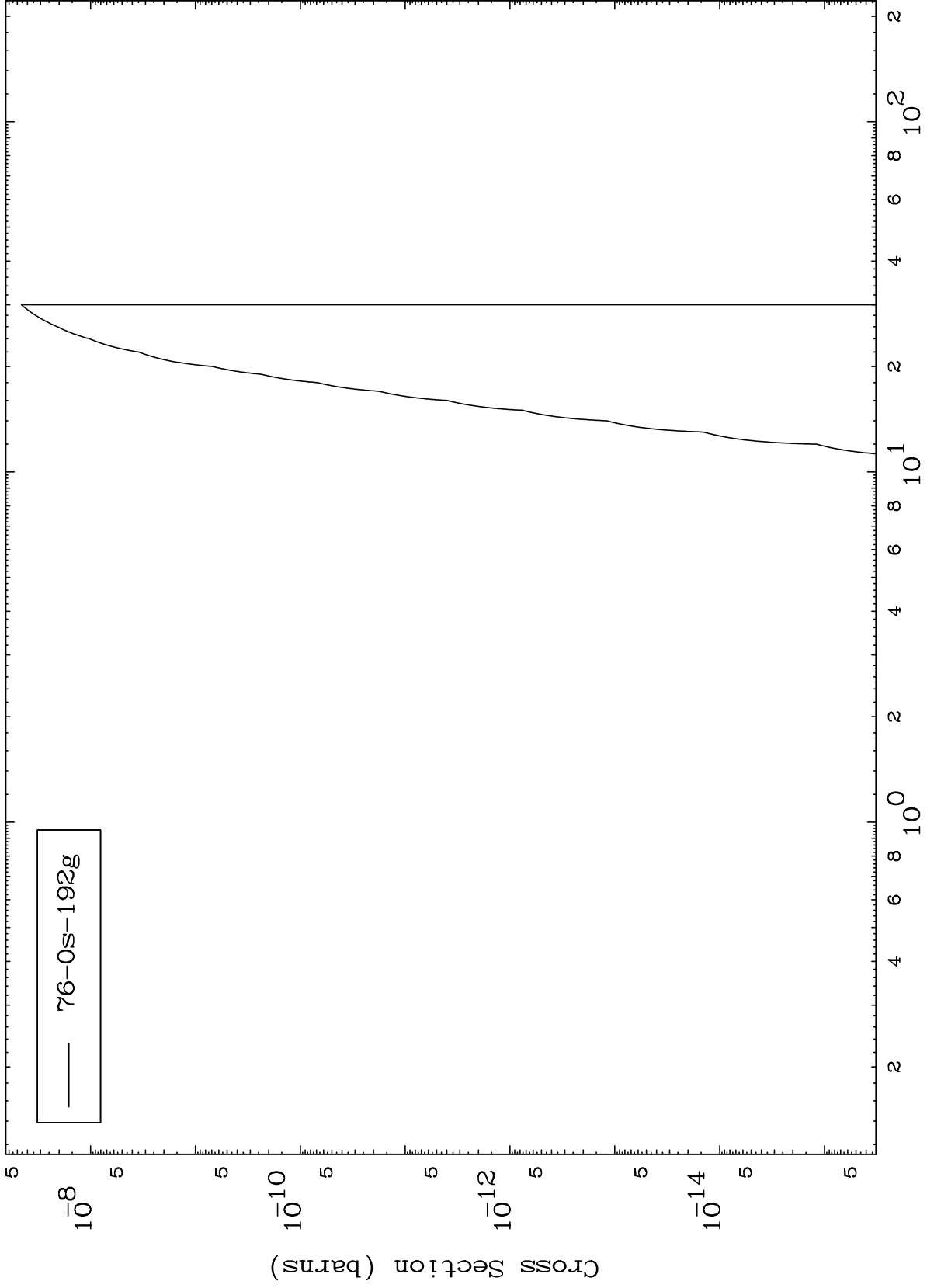
<sup>77</sup>Ir-194

MAT 7734

(n,p)  $\alpha$

<sup>77</sup>Ir-194

Radionuclide Production Cross Section



30

Incident Energy (MeV)

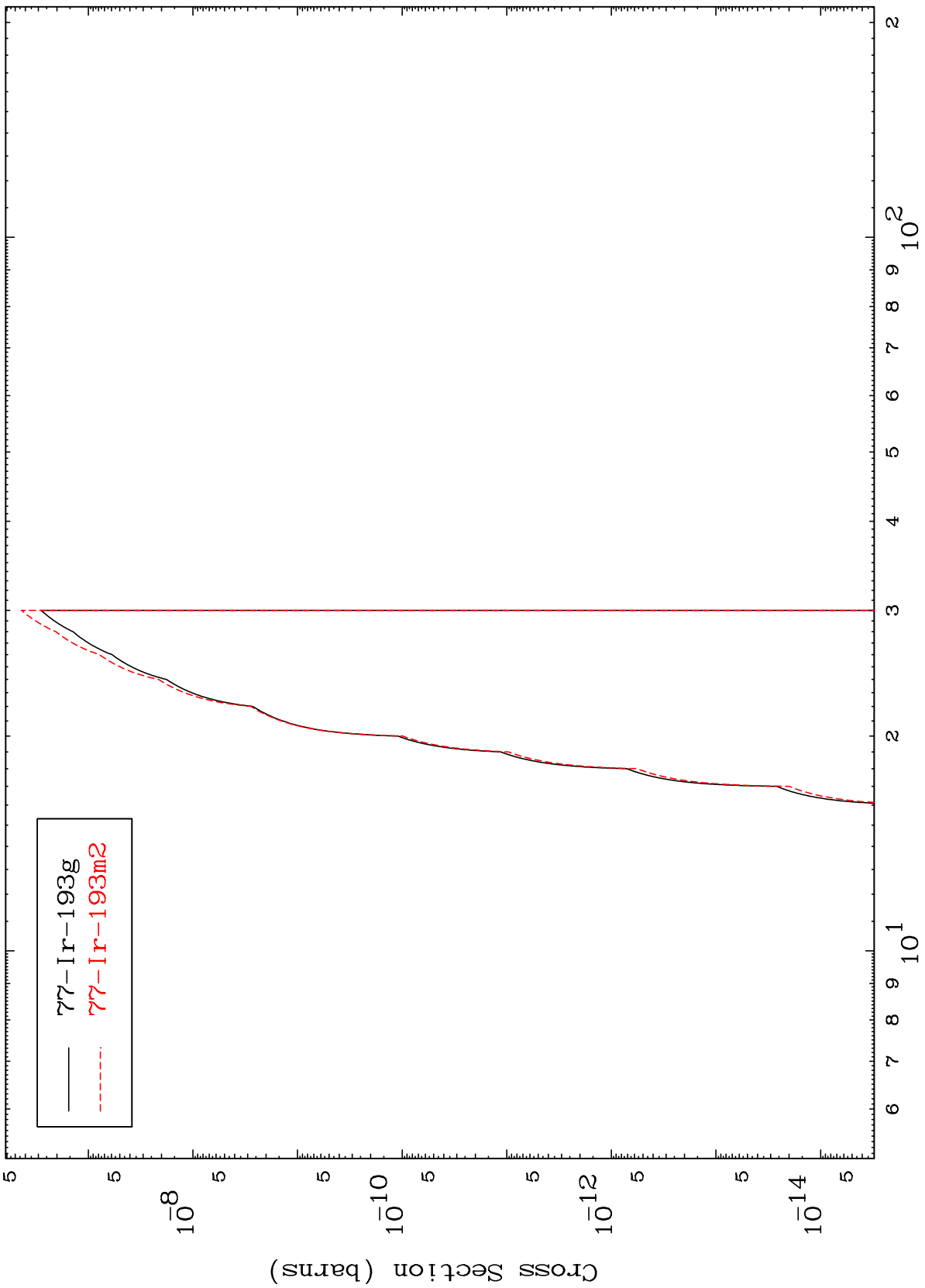
<sup>77</sup>Ir-194

MAT 7734

$^{77}\text{Ir}$  t

$^{77}\text{Ir-194}$

Radionuclide Production Cross Section

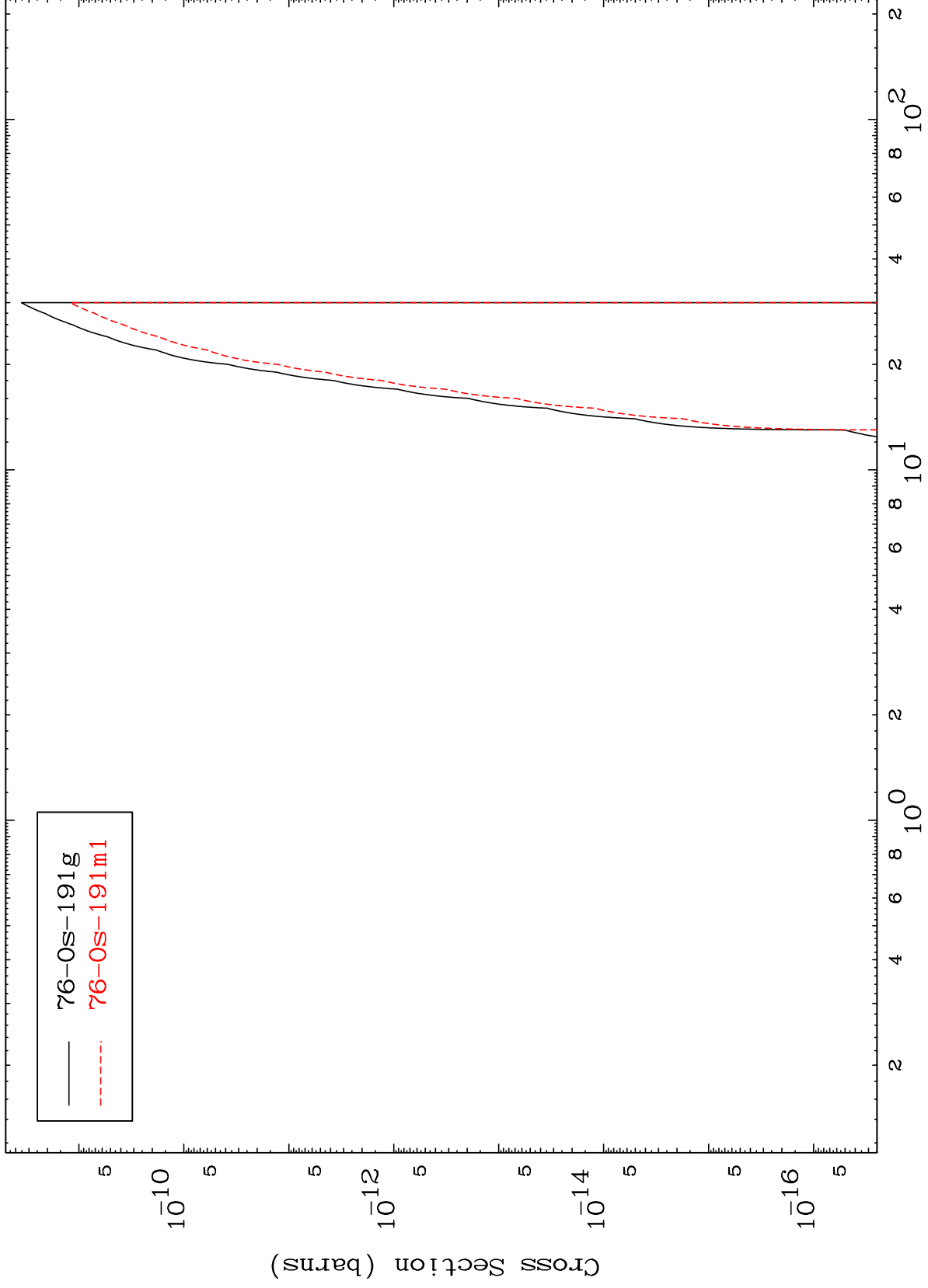


MAT 7734

(n,d)  $\alpha$

<sup>77</sup>Ir-194

Radionuclide Production Cross Section



32

Incident Energy (MeV)

<sup>77</sup>Ir-194