

Program EVALPLOT  
(Version 2021-1)

by

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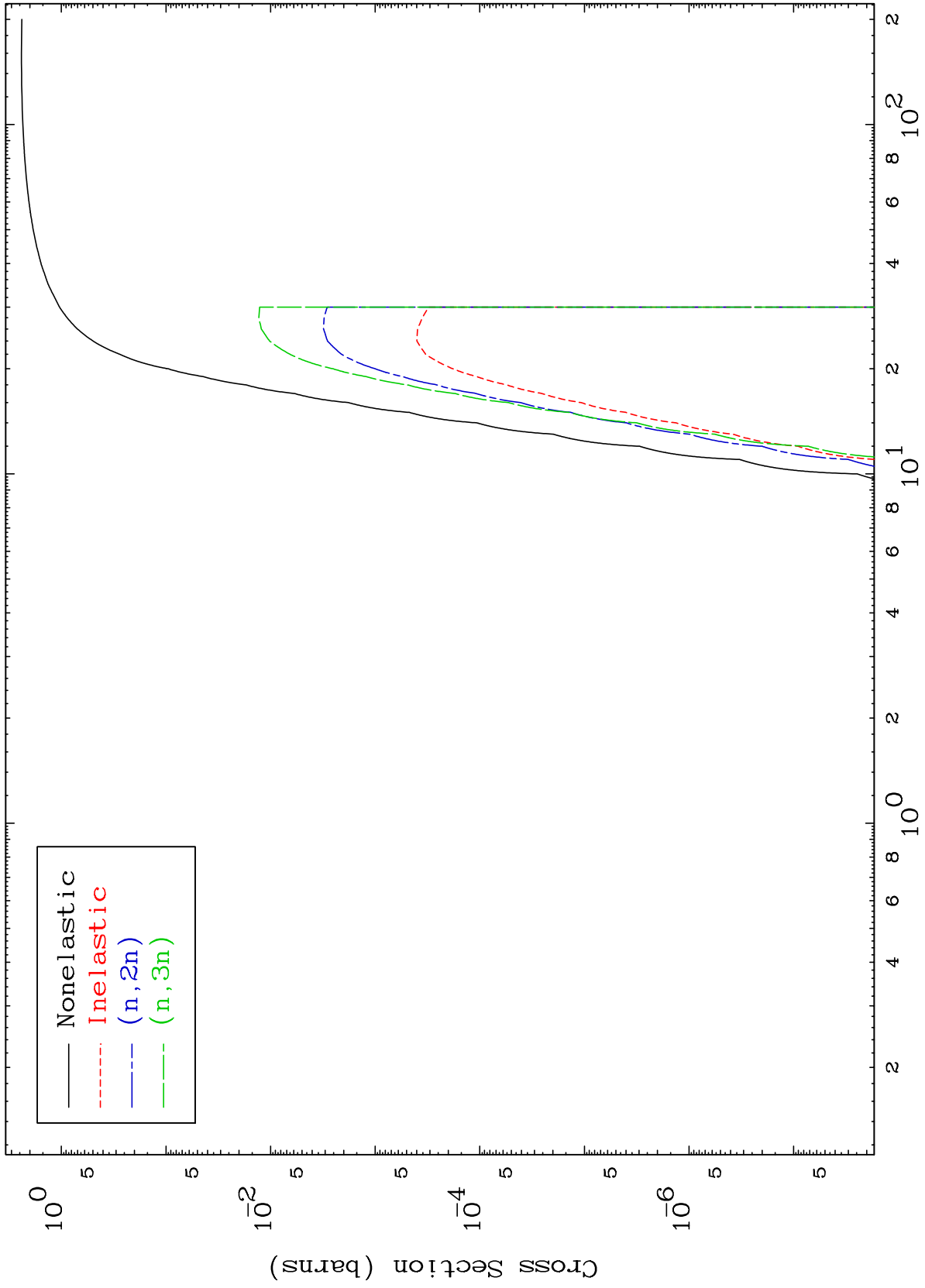
Press Mouse Button to Start

MAT 7740

He-3 Major

0 Kelvin Cross Sections

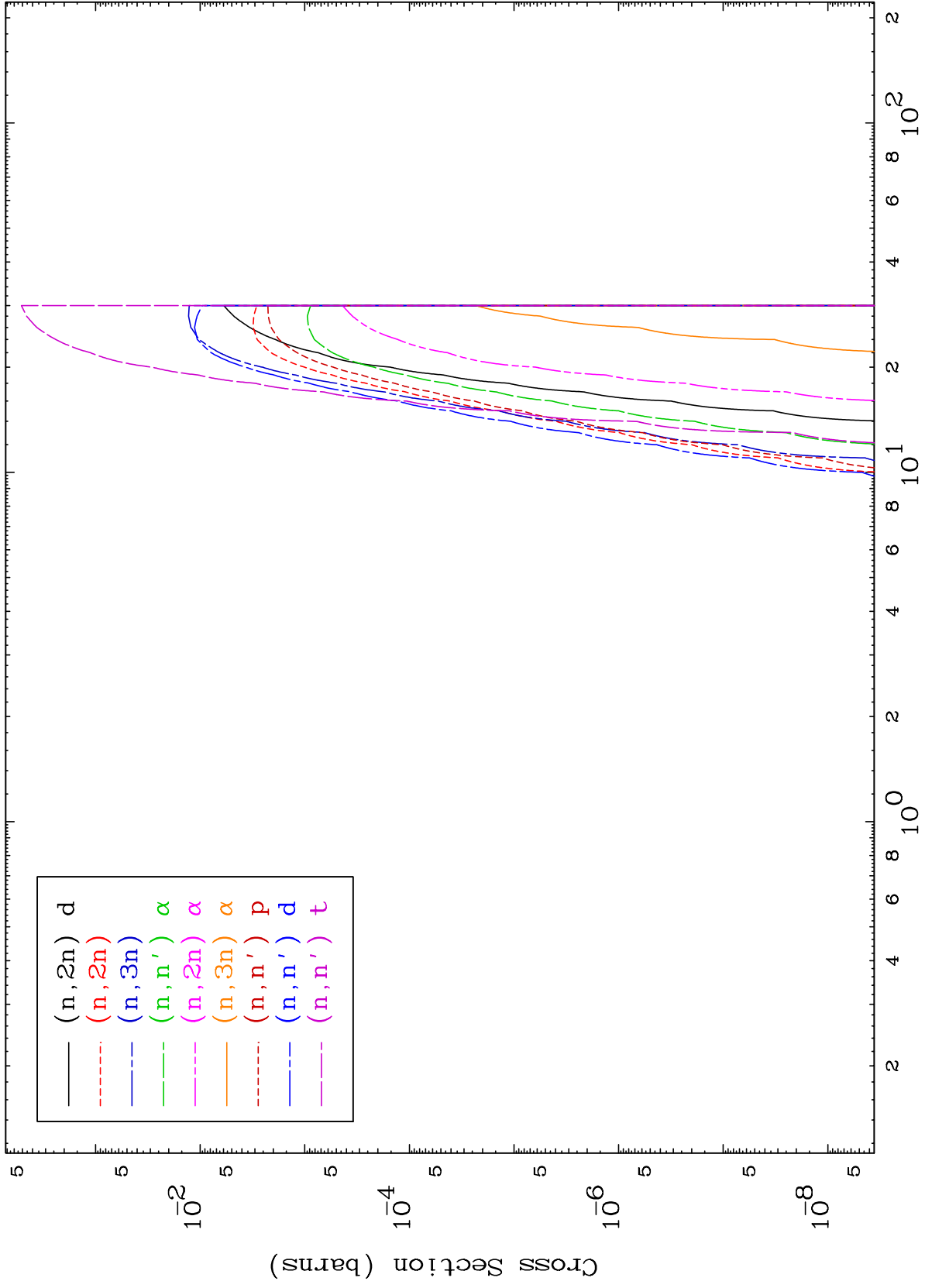
77-Ir-196



MAT 7740

He-3 Neutron Absorption  
0 Kelvin Cross Sections

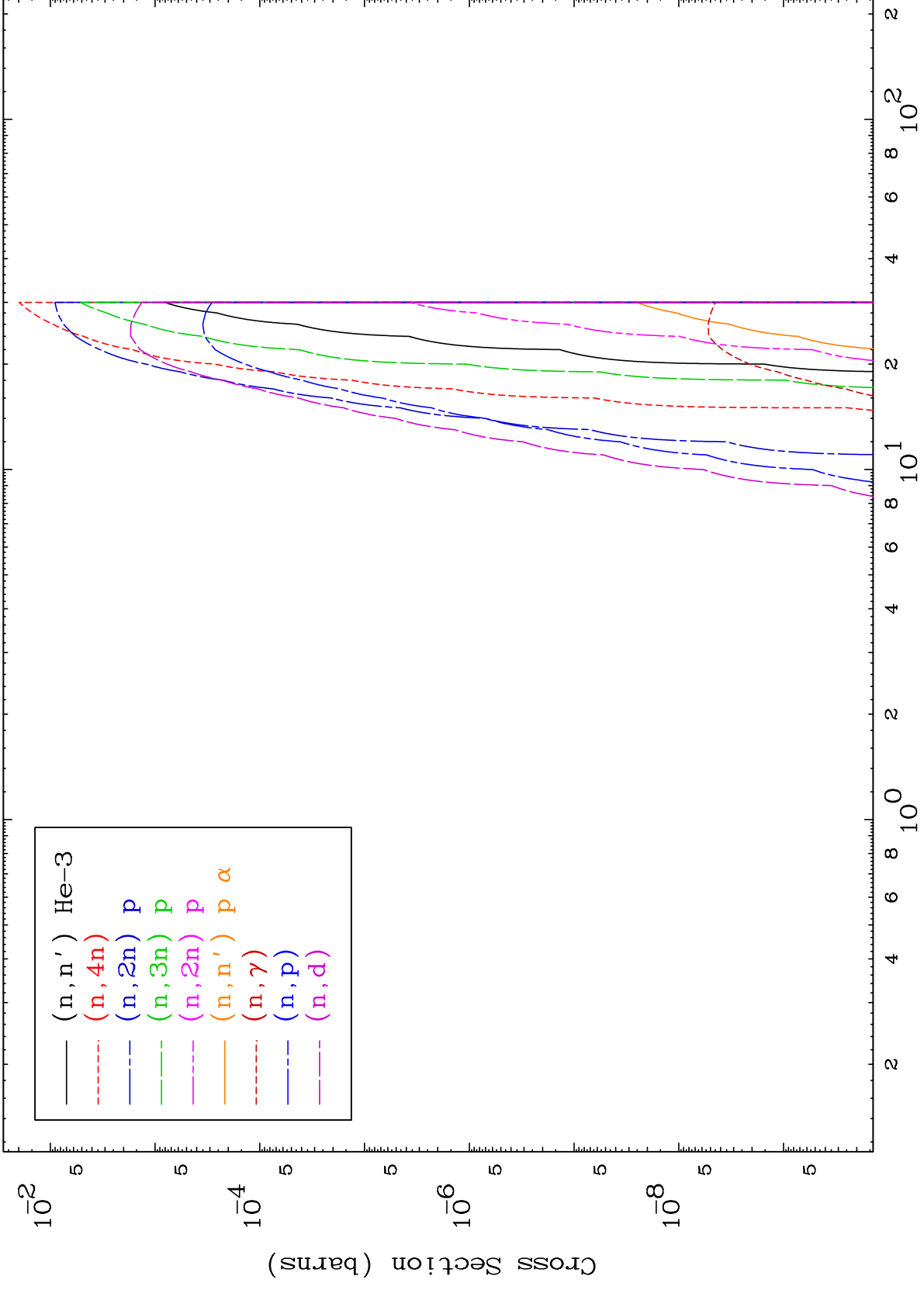
77-Ir-196



MAT 7740

He-3 Neutron Absorption  
0 Kelvin Cross Sections

77-Ir-196



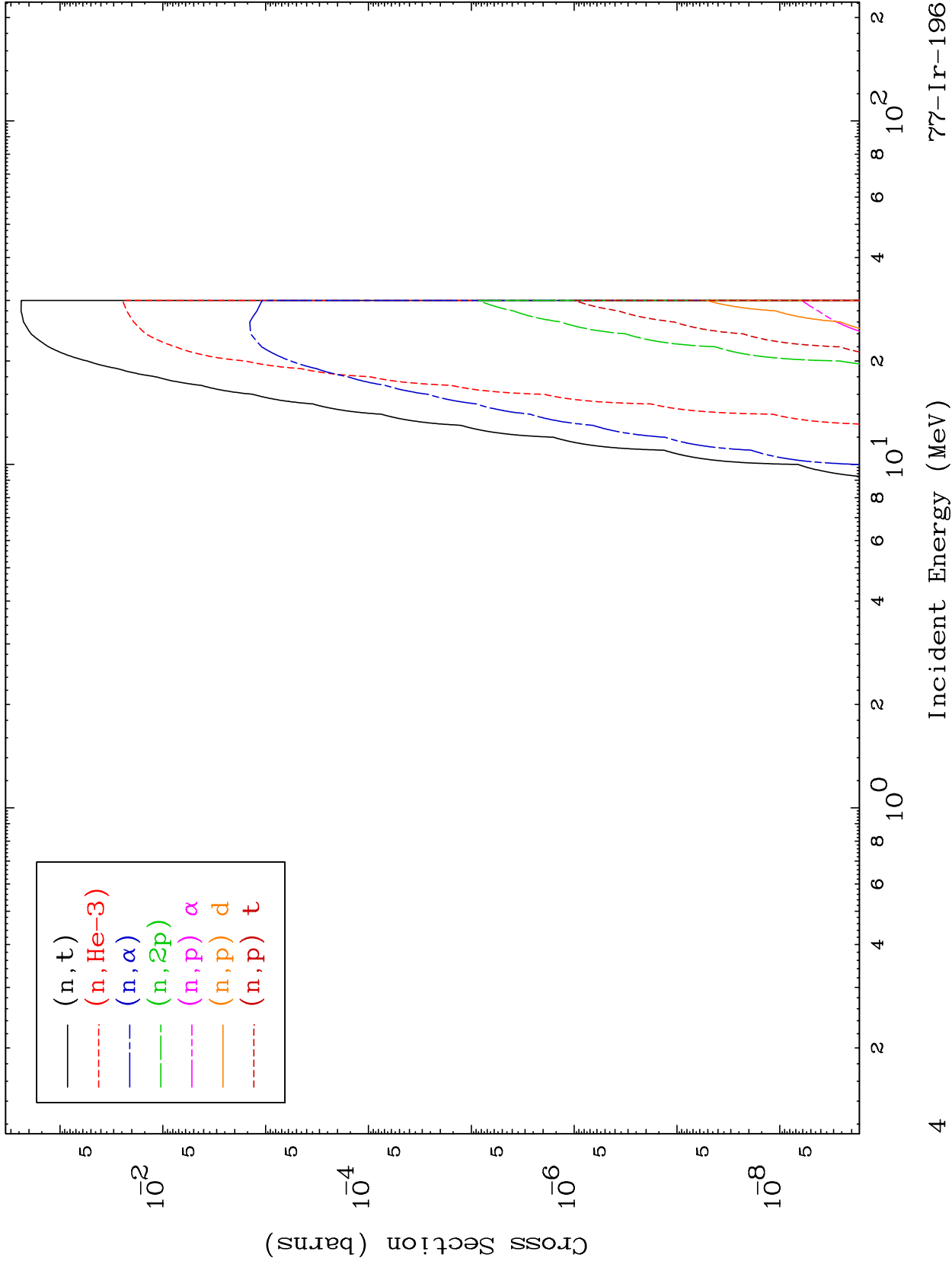
77-Ir-196

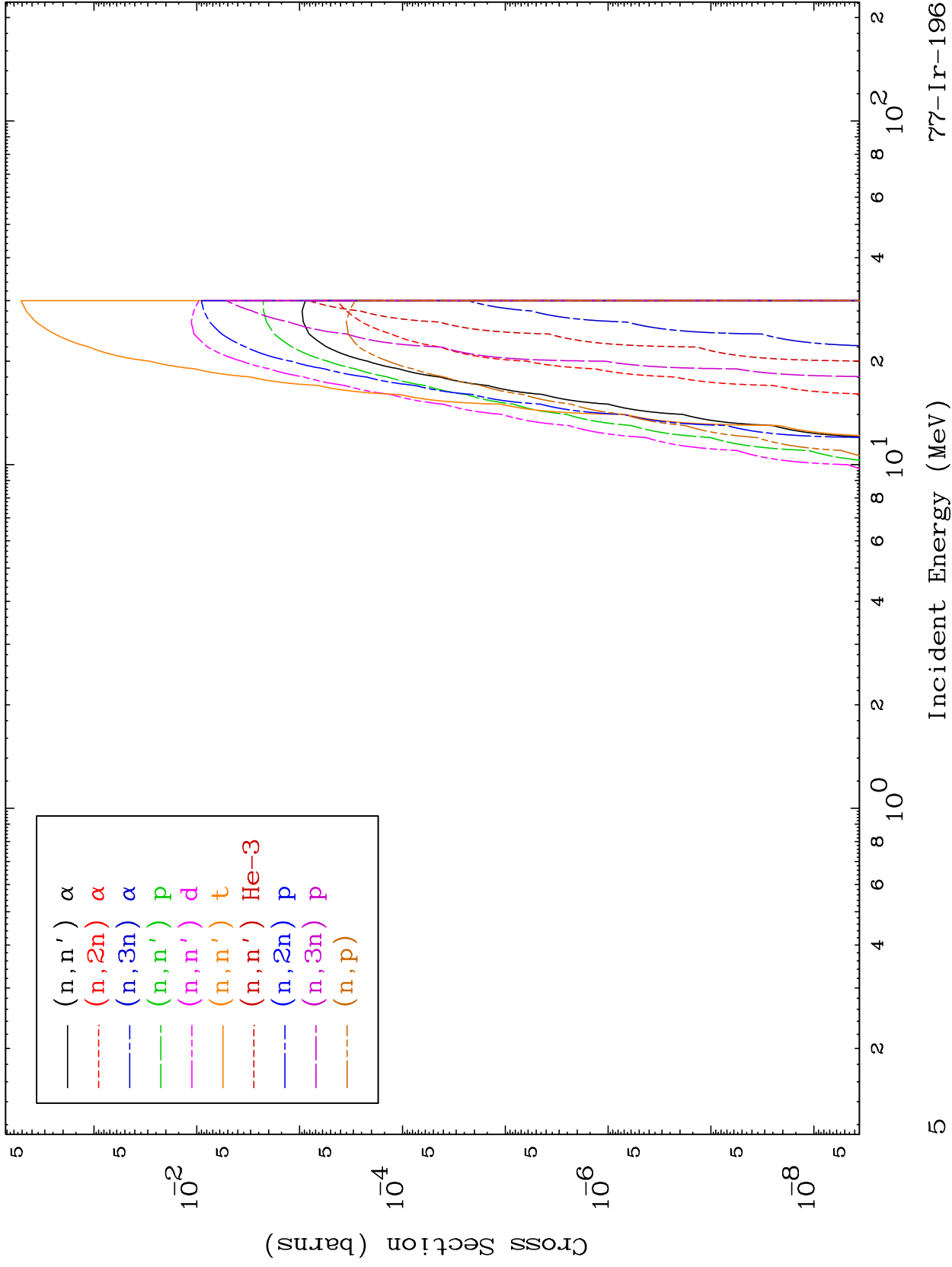
Incident Energy (MeV)

MAT 7740

He-3 Neutron Absorption  
0 Kelvin Cross Sections

77-Ir-196

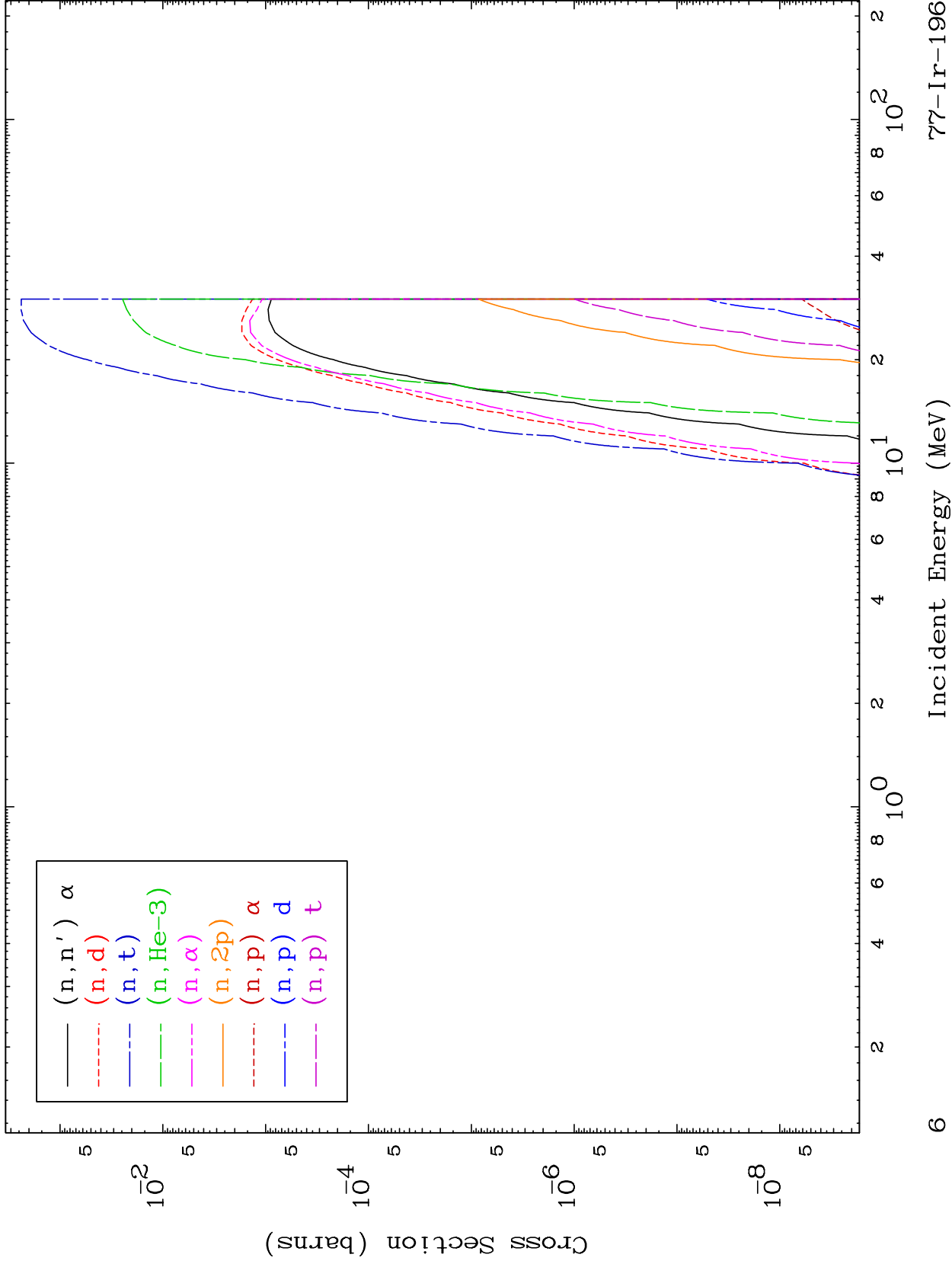




MAT 7740

He-3 Charged Particle  
0 Kelvin Cross Sections

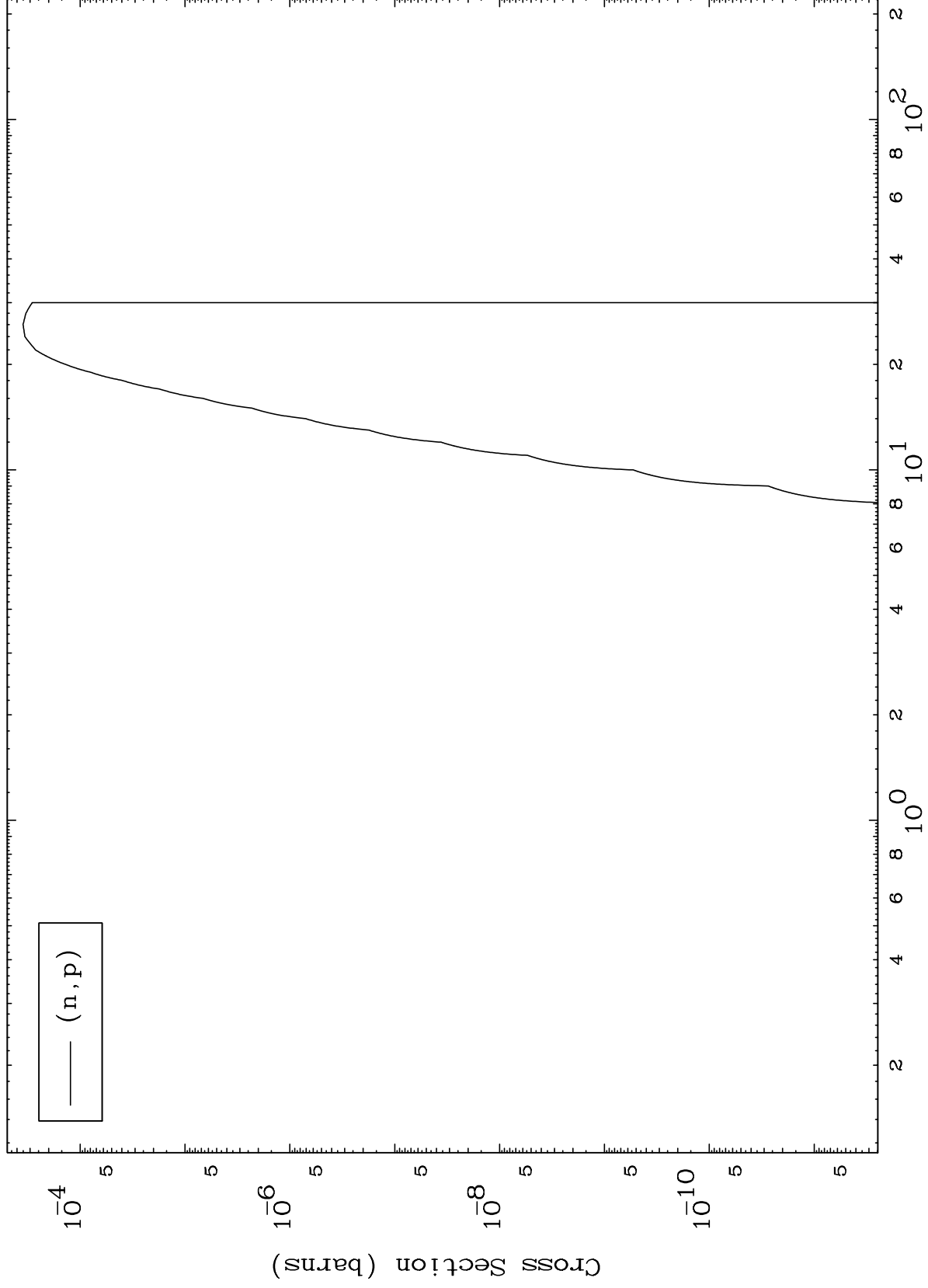
77-Ir-196



MAT 7740

77-Ir-196

(He-3,p) Levels  
0 Kelvin Cross Sections

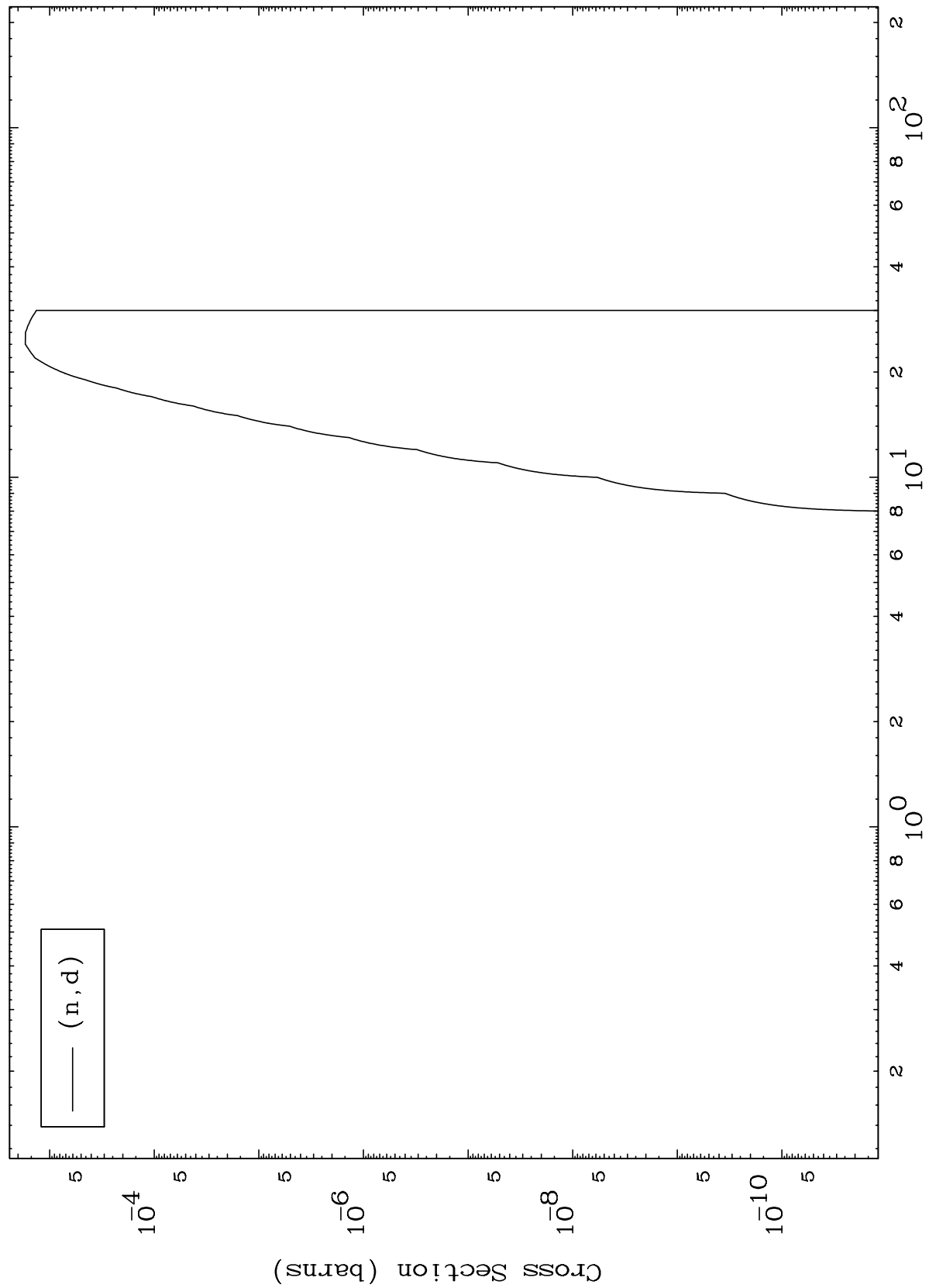


MAT 7740

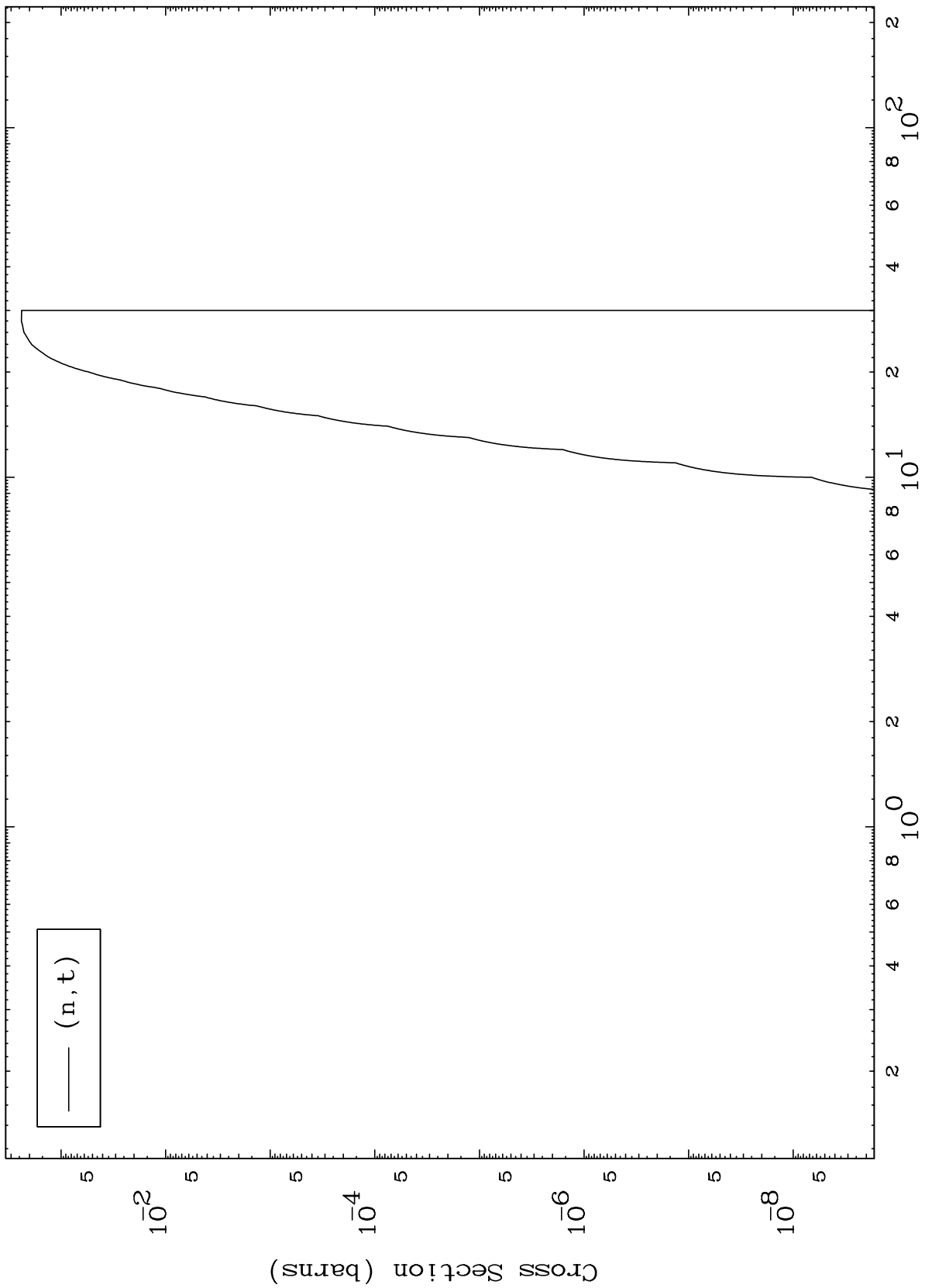
(He-3,d) Levels

77-Ir-196

0 Kelvin Cross Sections



0 Kelvin Cross Sections



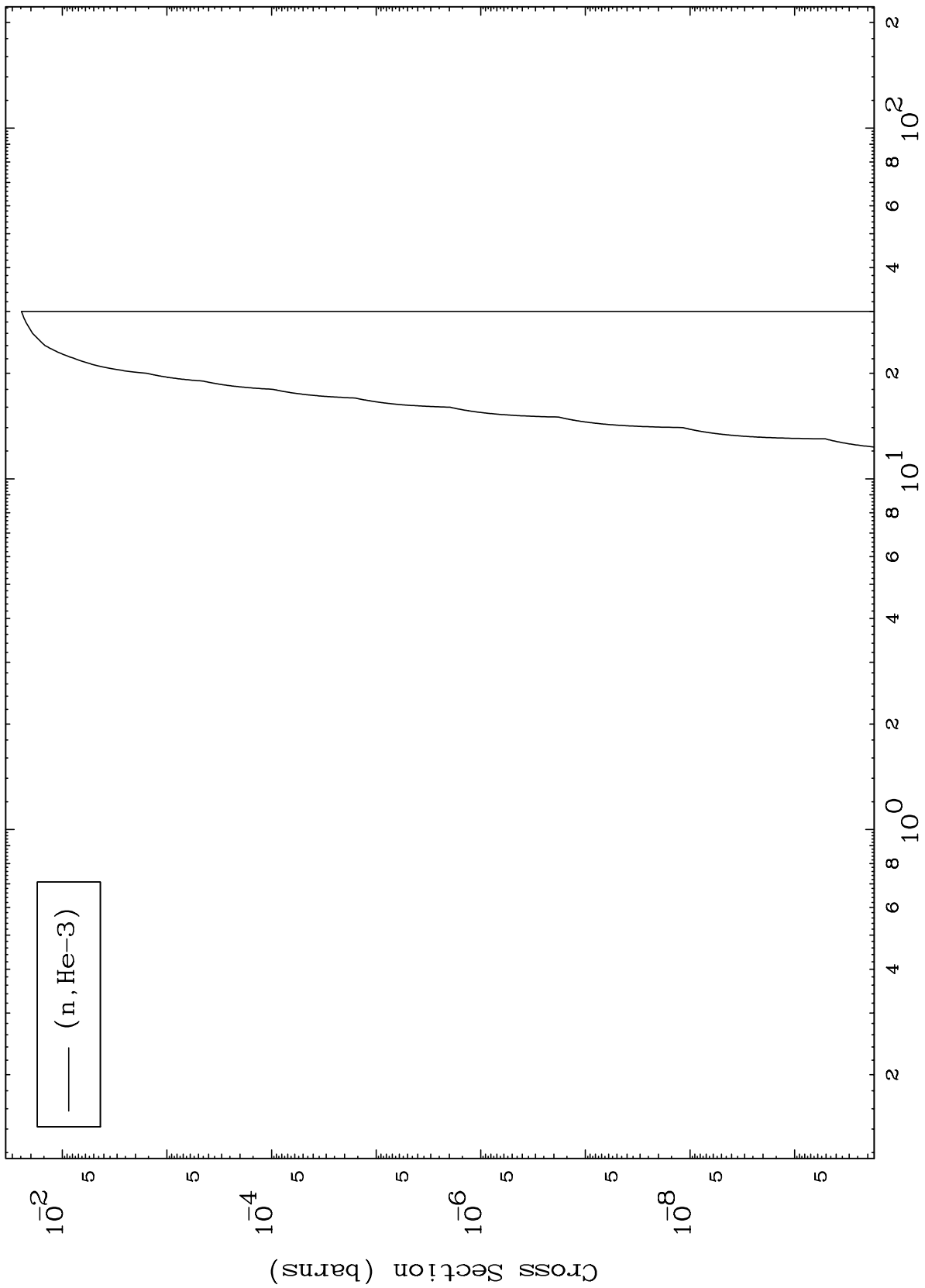
(n, t)

MAT 7740

(He-3, He3) Levels

77-Ir-196

0 Kelvin Cross Sections



10

Incident Energy (MeV)

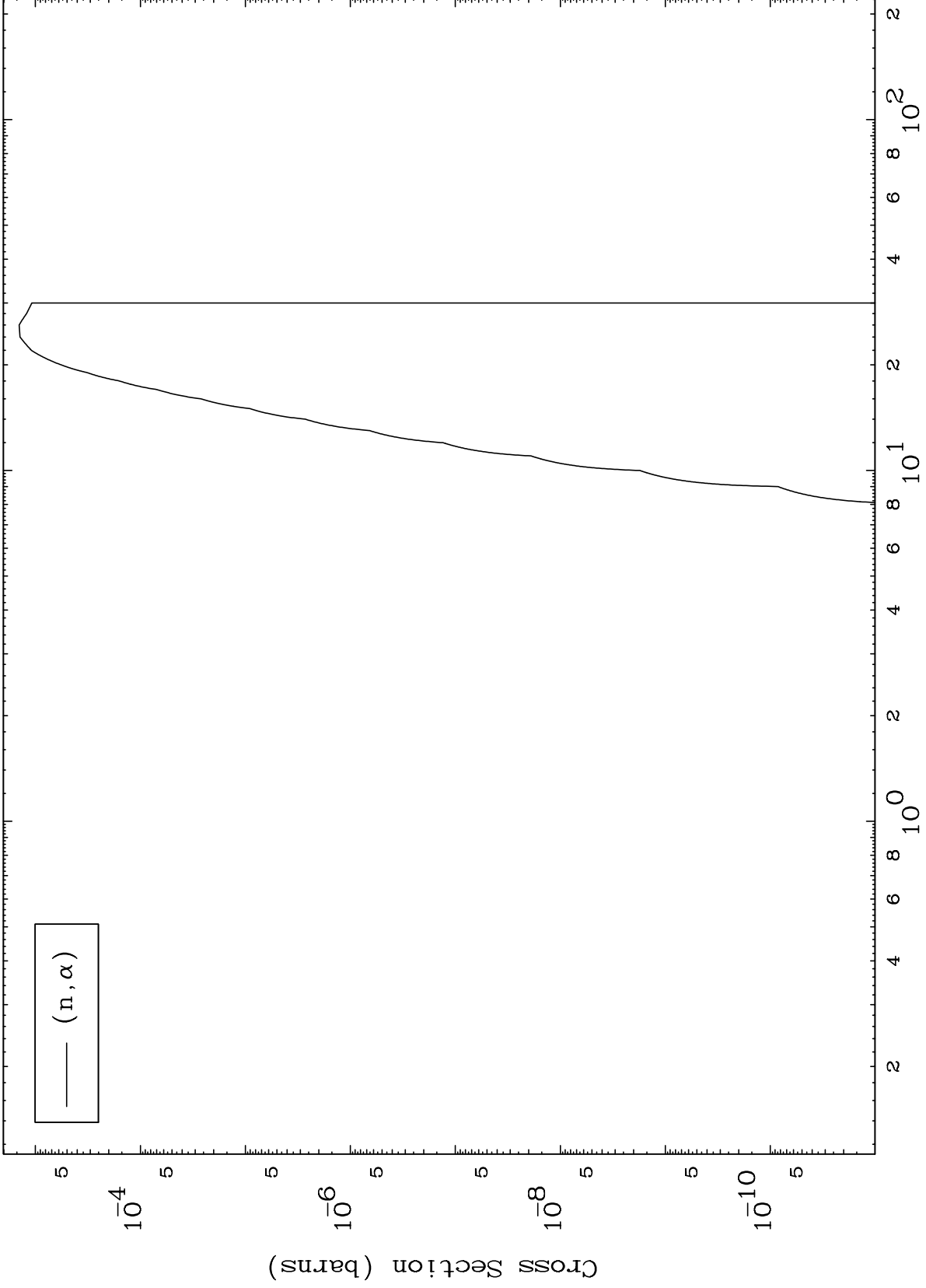
77-Ir-196

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(He-3,  $\alpha$ ) Levels

77-Ir-196

0 Kelvin Cross Sections

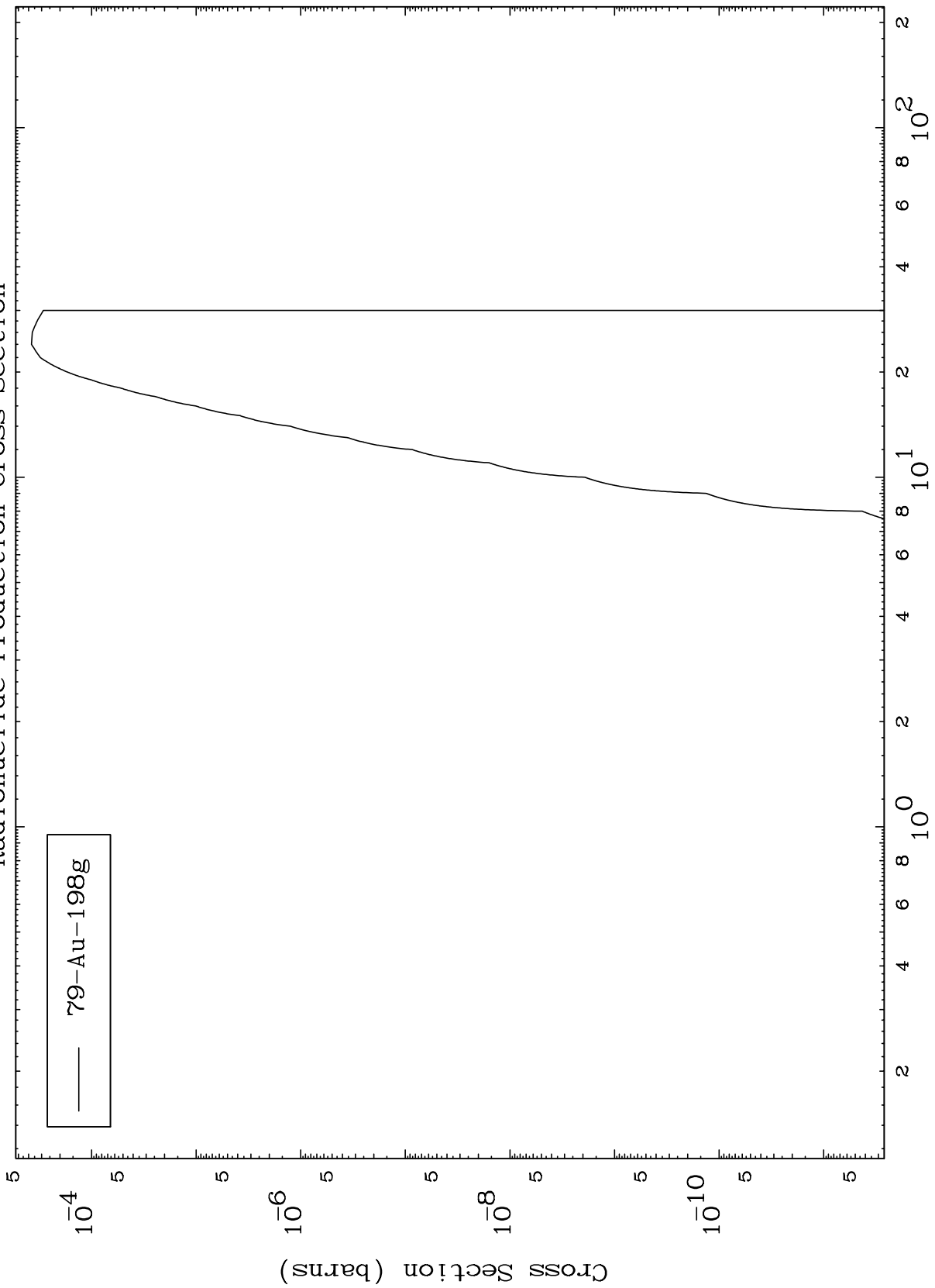


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Inelastic

77-Ir-196

Radionuclide Production Cross Section

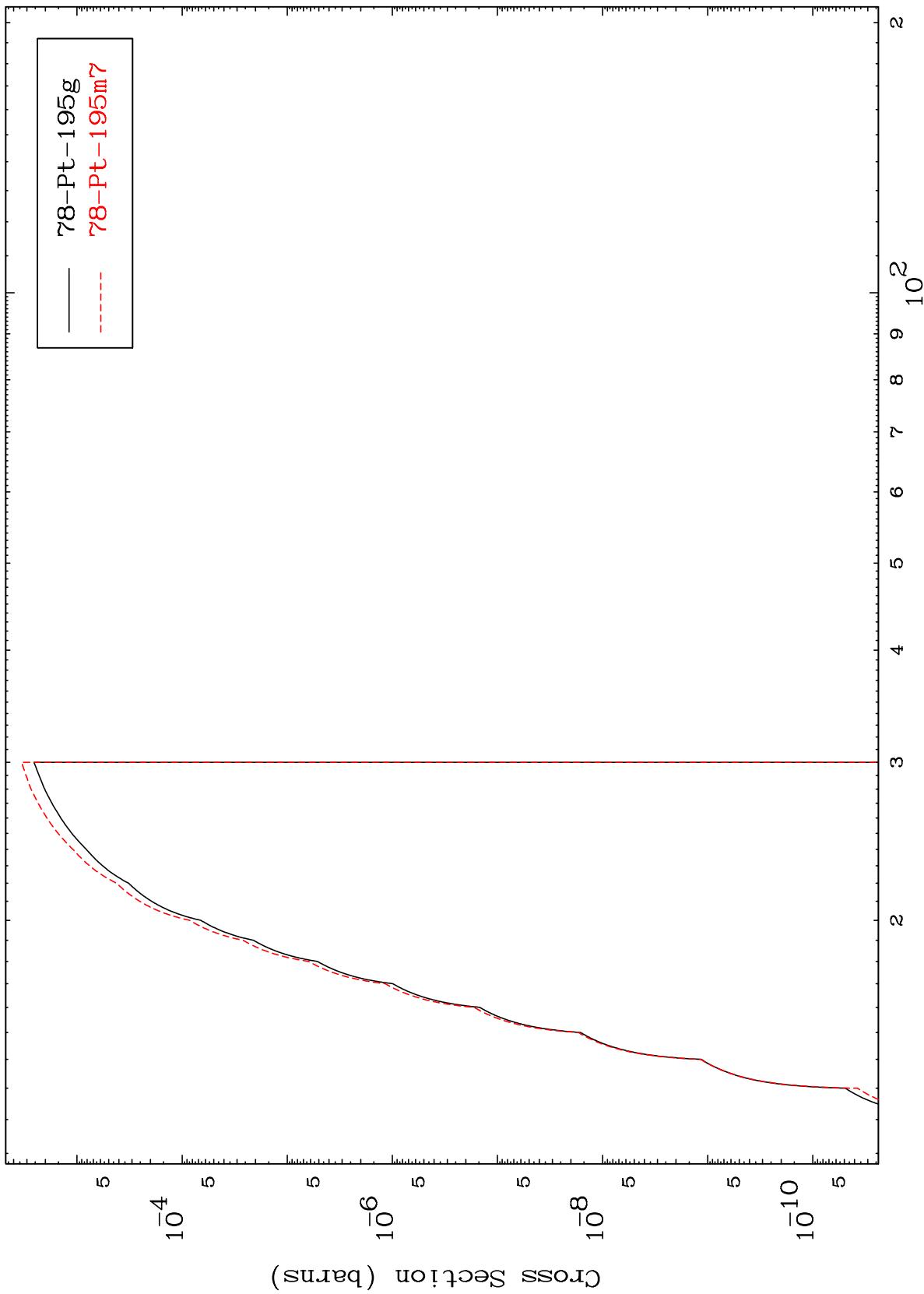


12

Incident Energy (MeV)

77-Ir-196

Radionuclide Production Cross Section

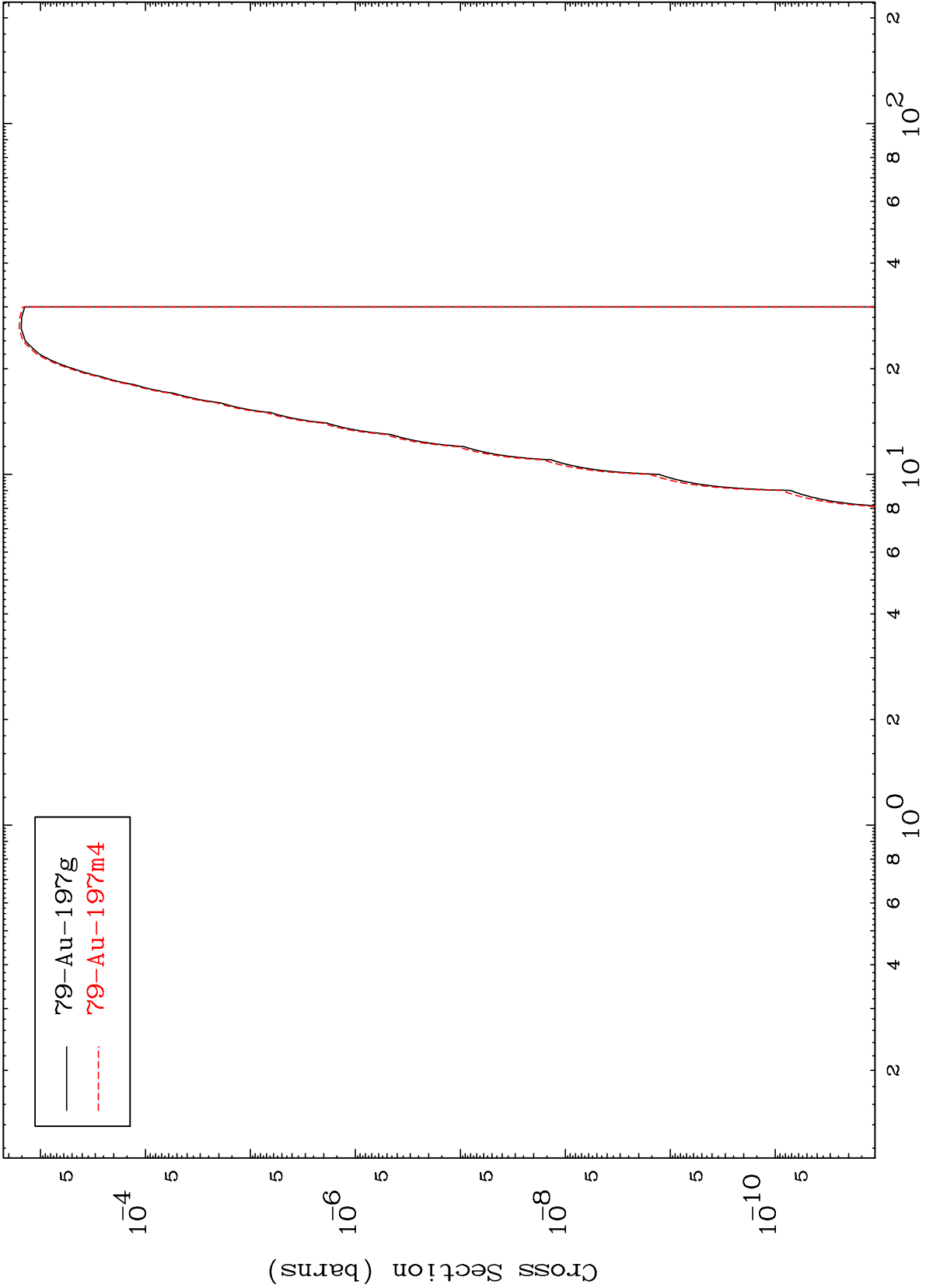


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(n,2n)

77-Ir-196

Radionuclide Production Cross Section



14

Incident Energy (MeV)

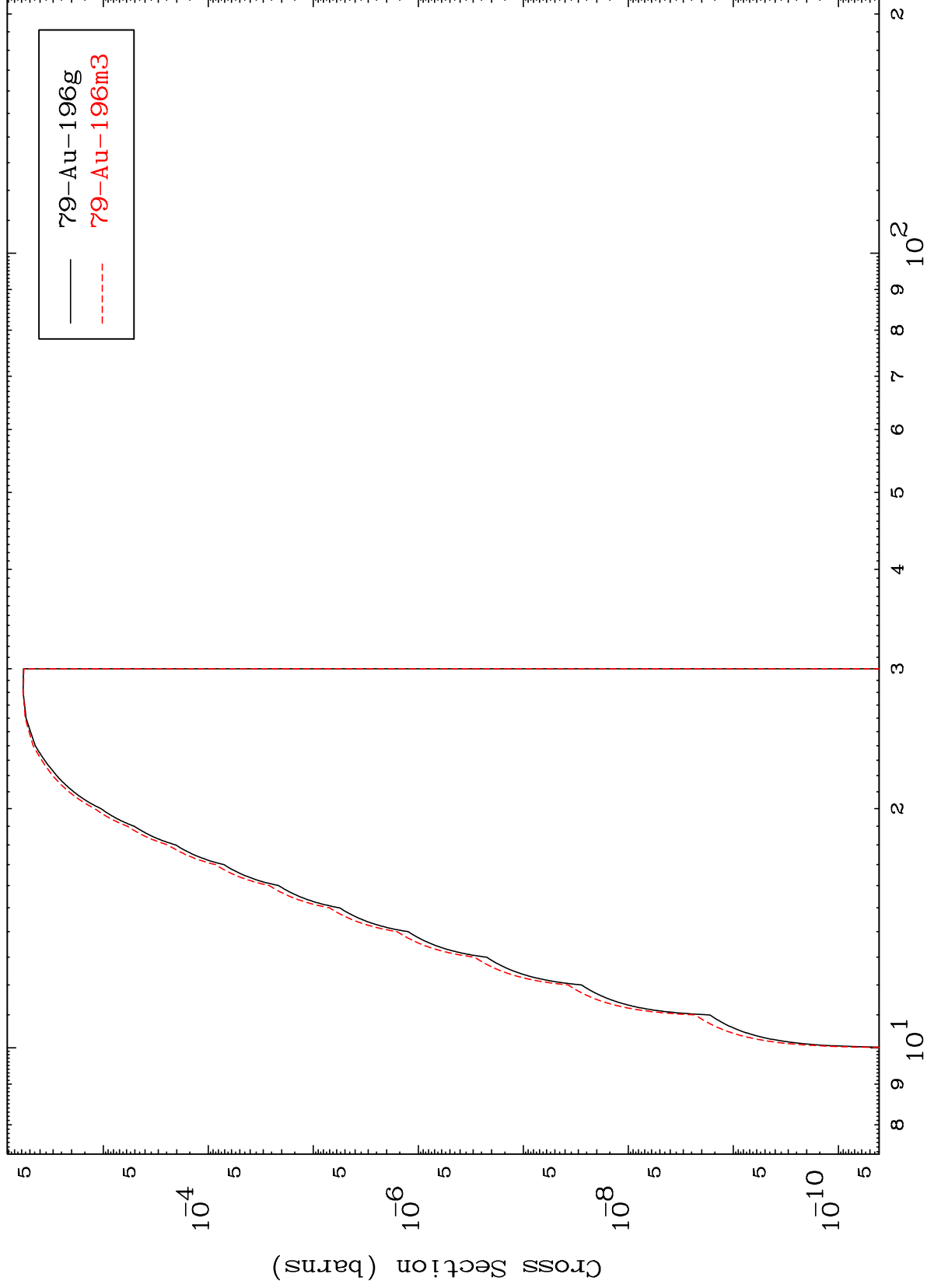
77-Ir-196

MAT 7740

(n,3n)

77-Ir-196

Radionuclide Production Cross Section



15

Incident Energy (MeV)

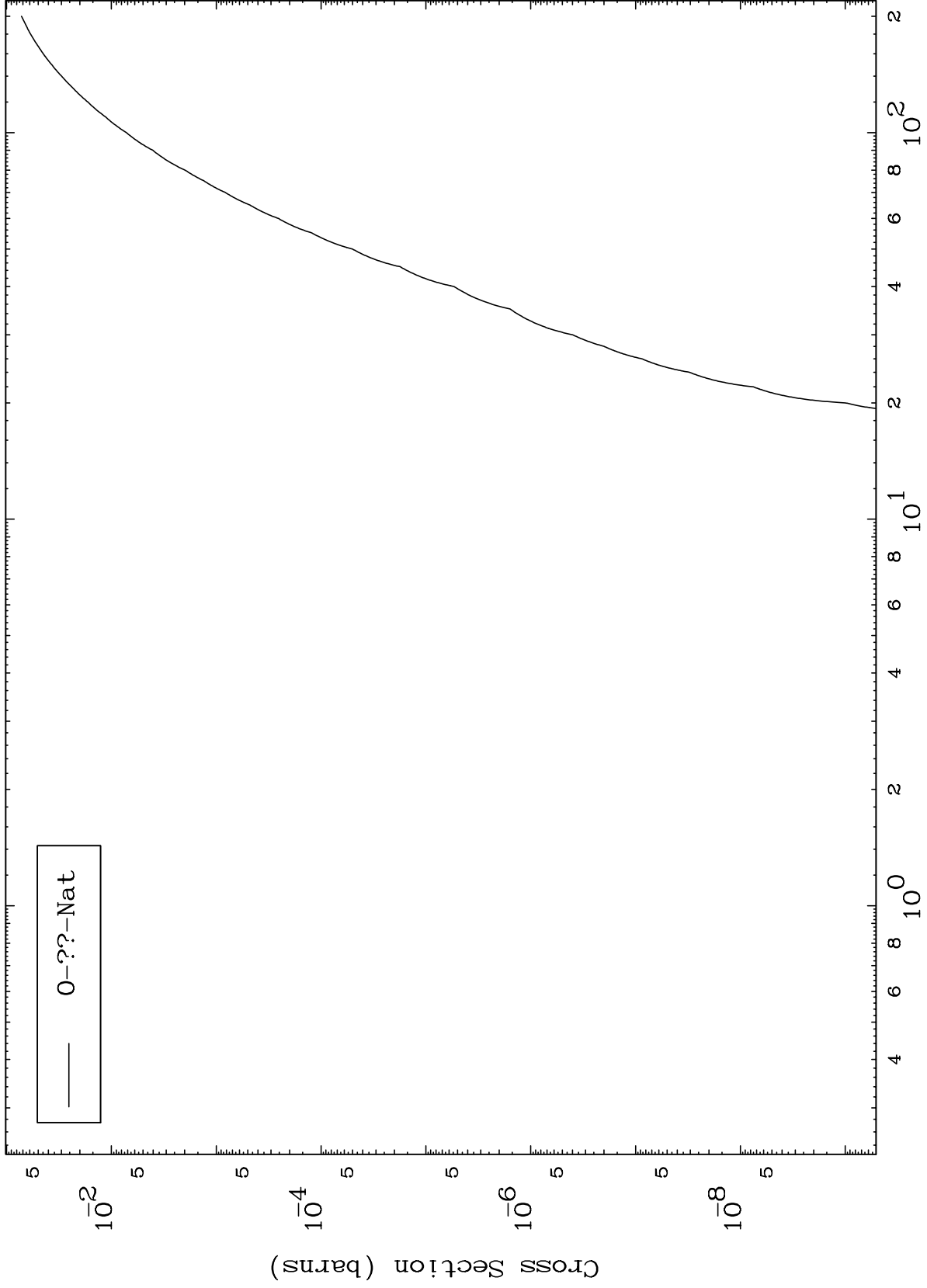
77-Ir-196

MAT 7740

Fission

<sup>77</sup>Ir-196

Radionuclide Production Cross Section



16

Incident Energy (MeV)

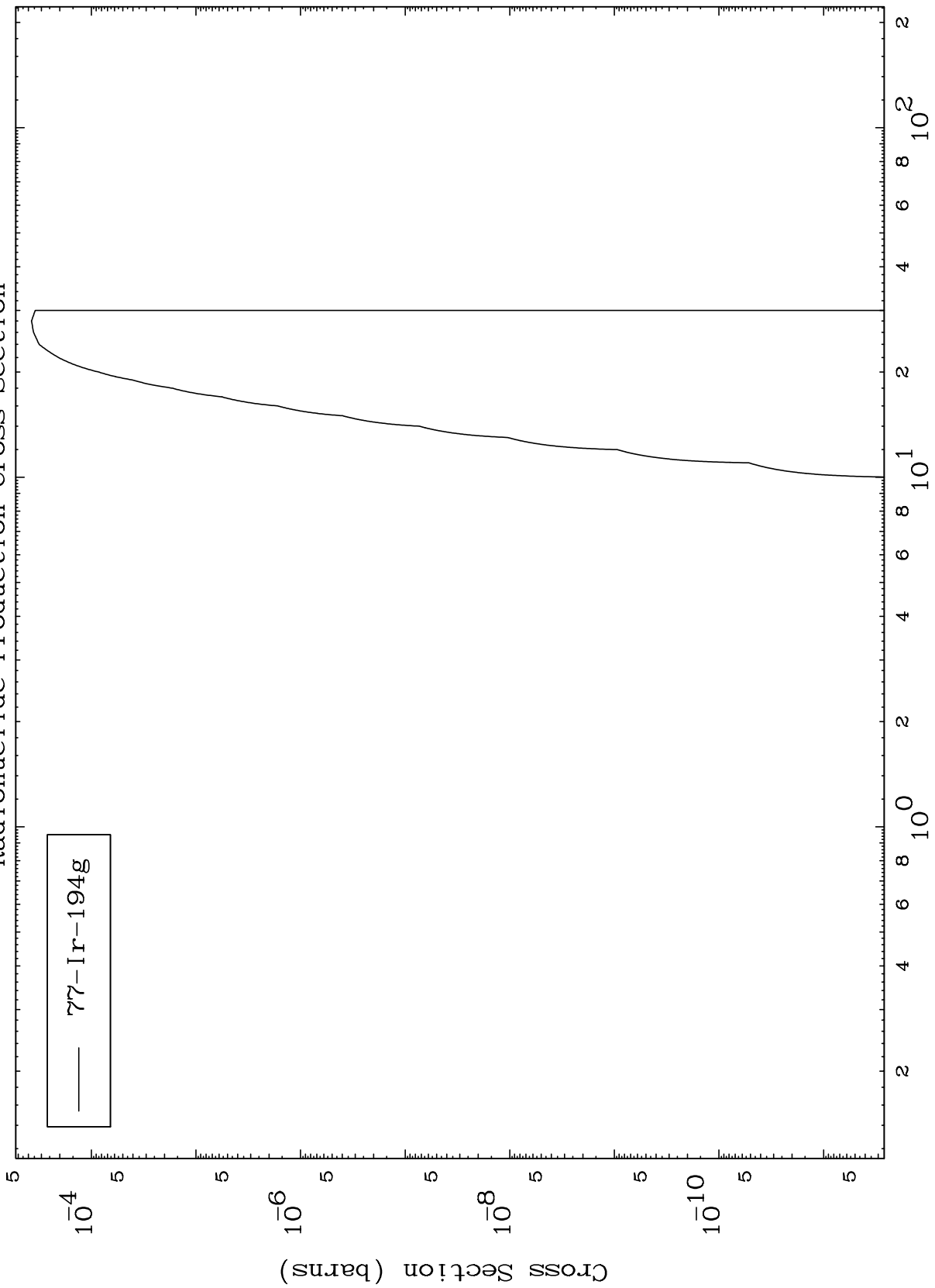
<sup>77</sup>Ir-196

MAT 7740

$(n, n') \alpha$

<sup>77</sup>Ir-196

Radionuclide Production Cross Section



17

Incident Energy (MeV)

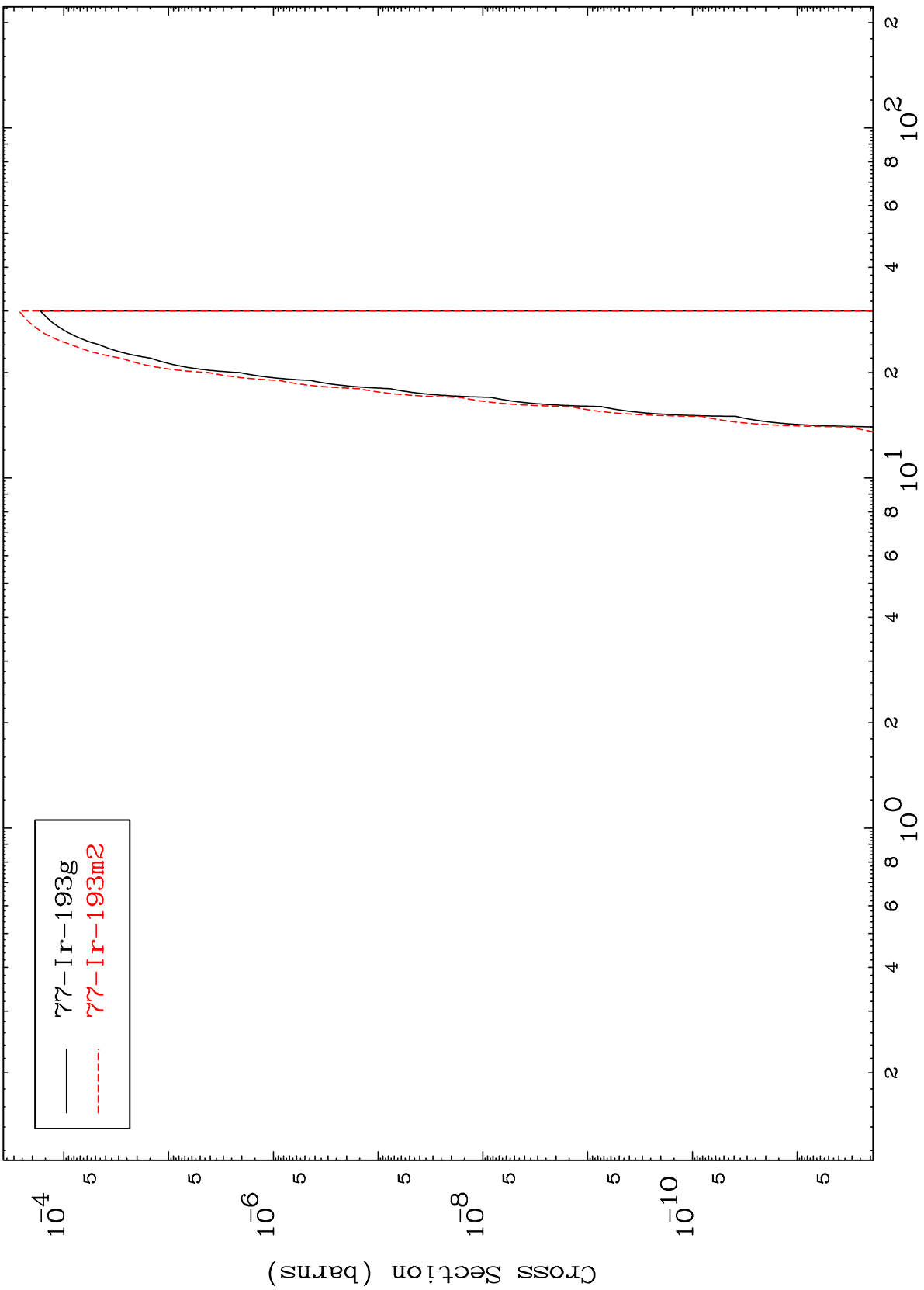
<sup>77</sup>Ir-196

MAT 7740

(n,2n)  $\alpha$

<sup>77</sup>Ir-196

Radionuclide Production Cross Section

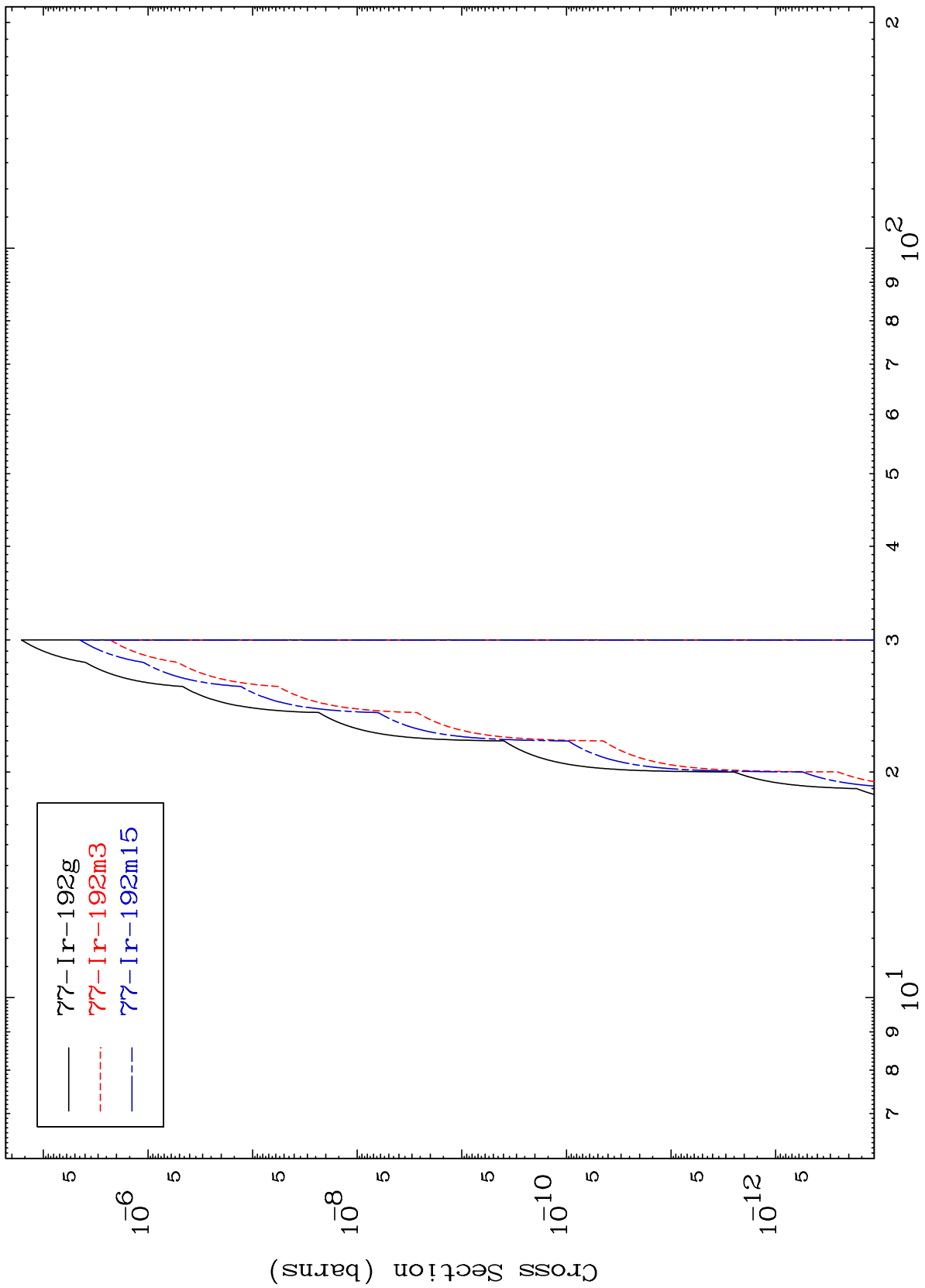


MAT 7740

$(n,3n) \alpha$

$^{77}\text{Ir-196}$

Radionuclide Production Cross Section



19

Incident Energy (MeV)

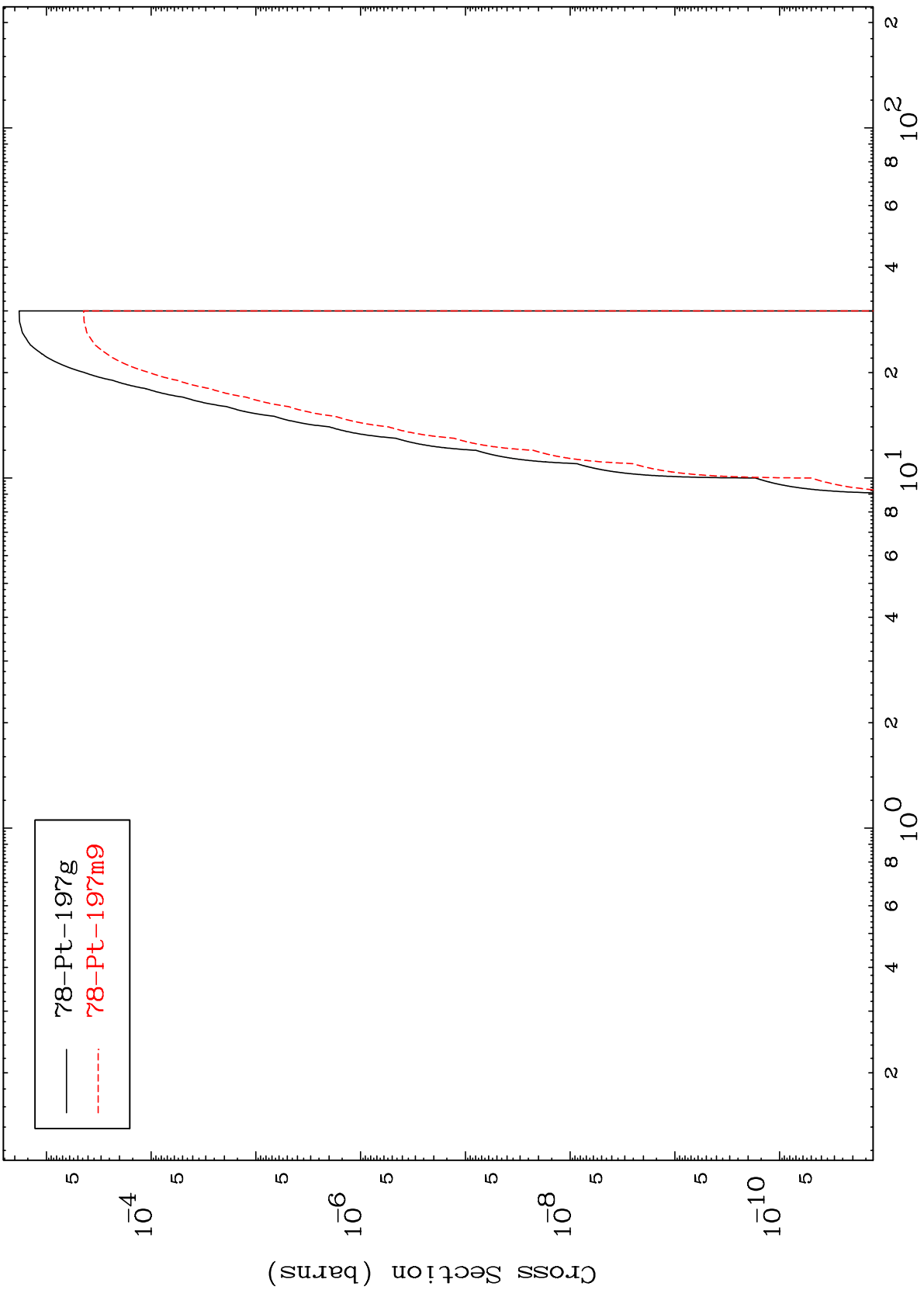
$^{77}\text{Ir-196}$

MAT 7740

(n,n') p

77-Ir-196

Radionuclide Production Cross Section



20

Incident Energy (MeV)

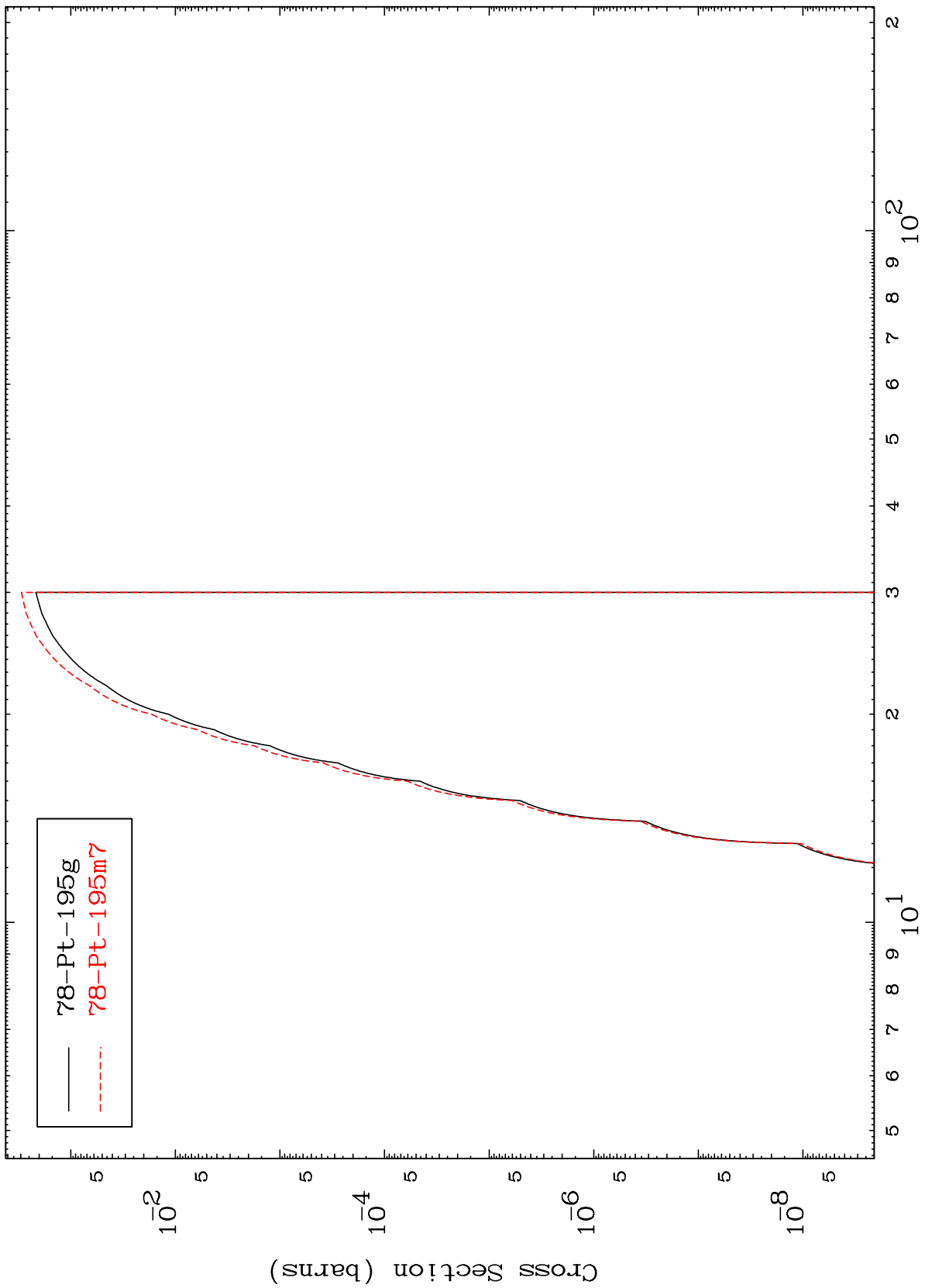
77-Ir-196

MAT 7740

(n,n') t

77-Ir-196

Radionuclide Production Cross Section

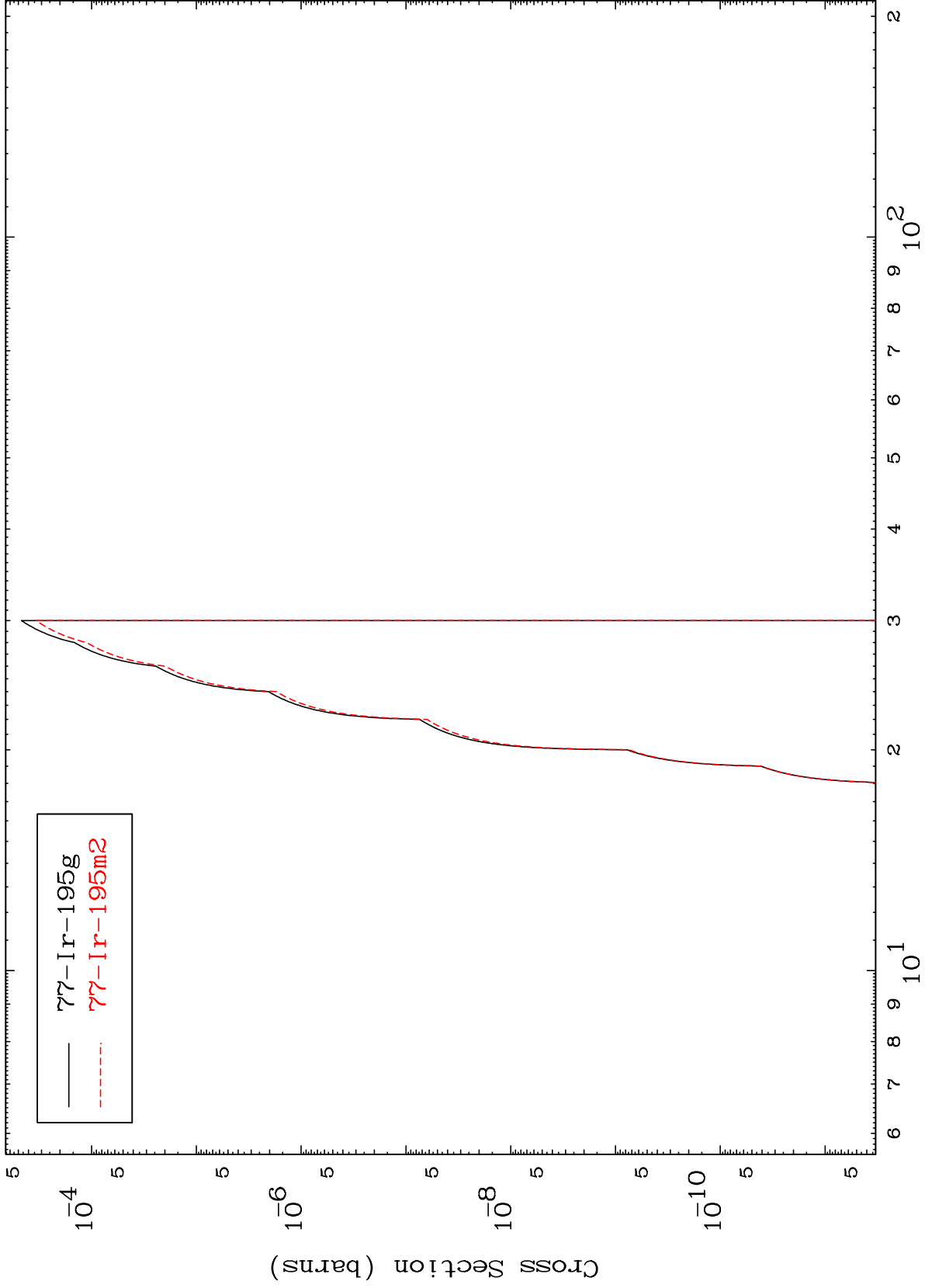


MAT 7740

(n,n') He-3

77-Ir-196

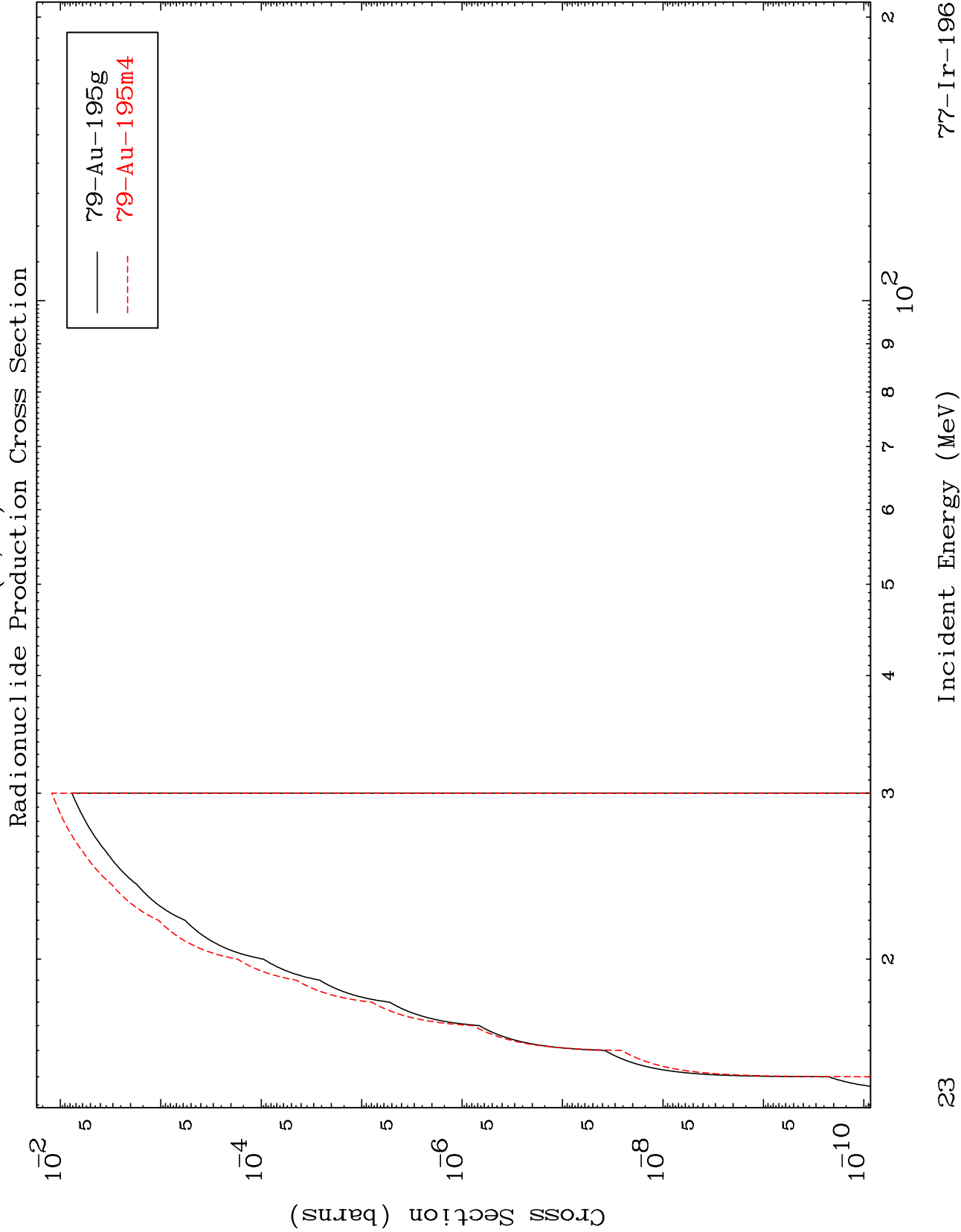
Radionuclide Production Cross Section



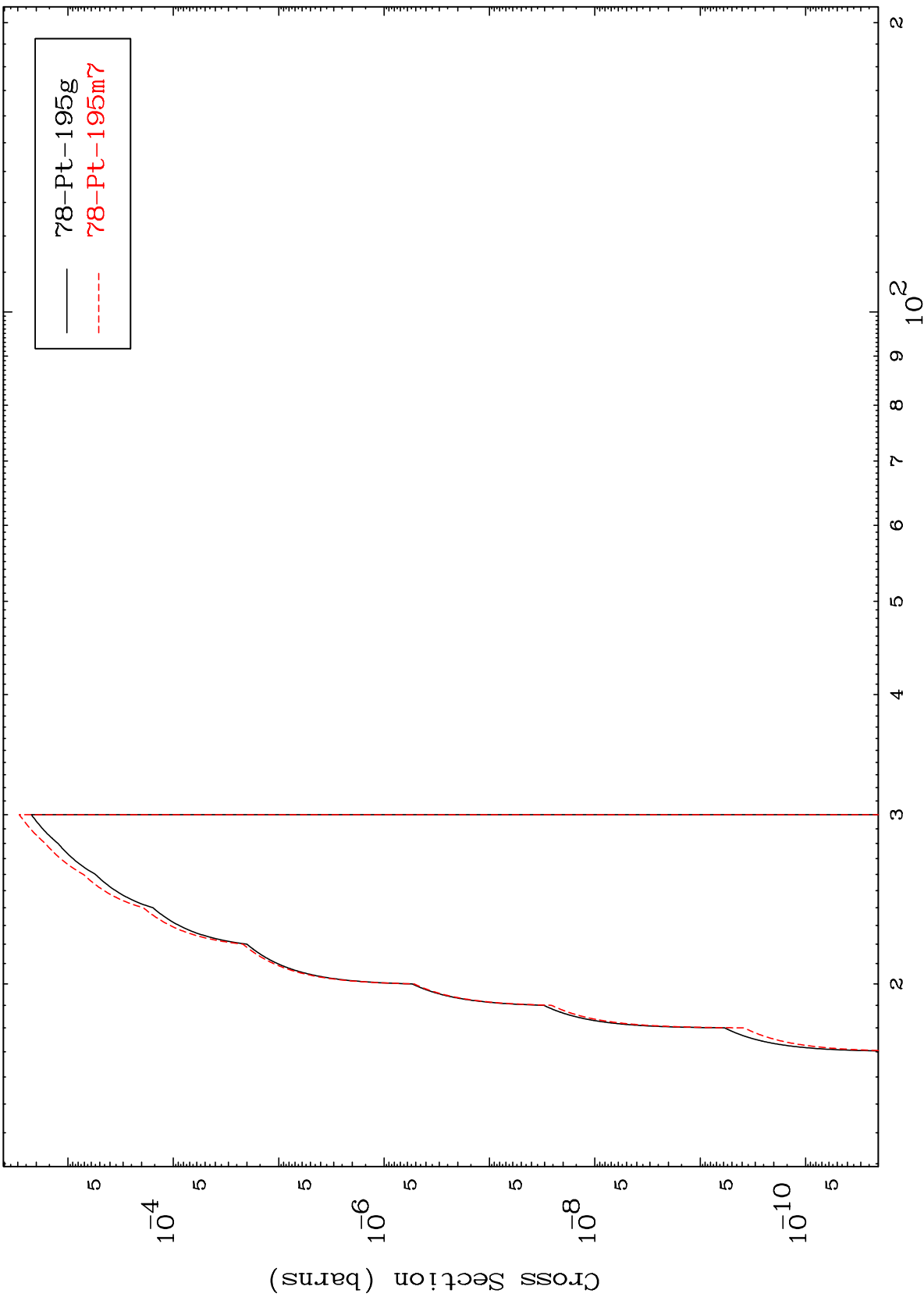
22

Incident Energy (MeV)

77-Ir-196



Radionuclide Production Cross Section

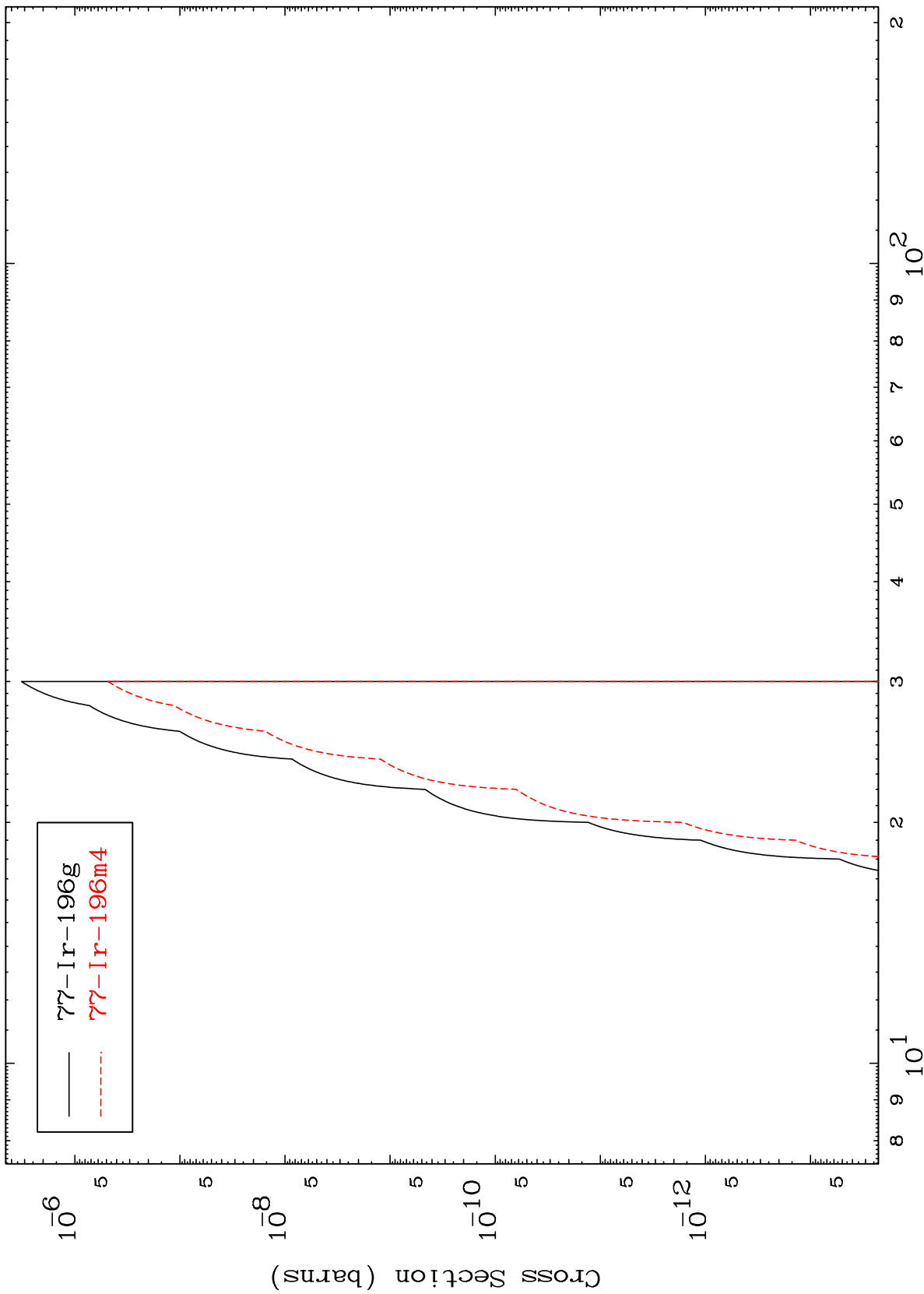


MAT 7740

(n,2n) p

<sup>77</sup>Ir-196

Radionuclide Production Cross Section

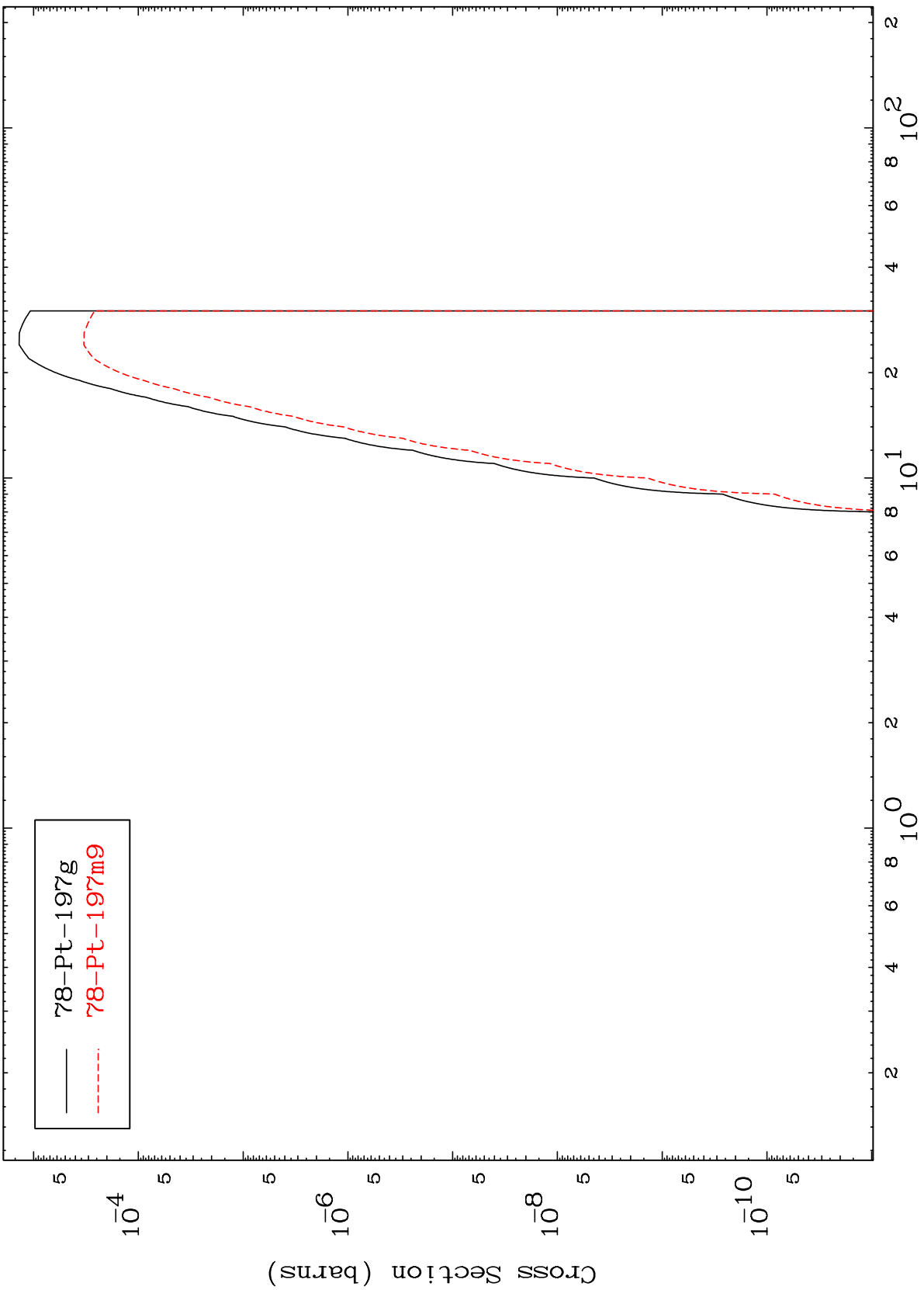


MAT 7740

(n,d)

<sup>77</sup>Ir-196

Radionuclide Production Cross Section



26

Incident Energy (MeV)

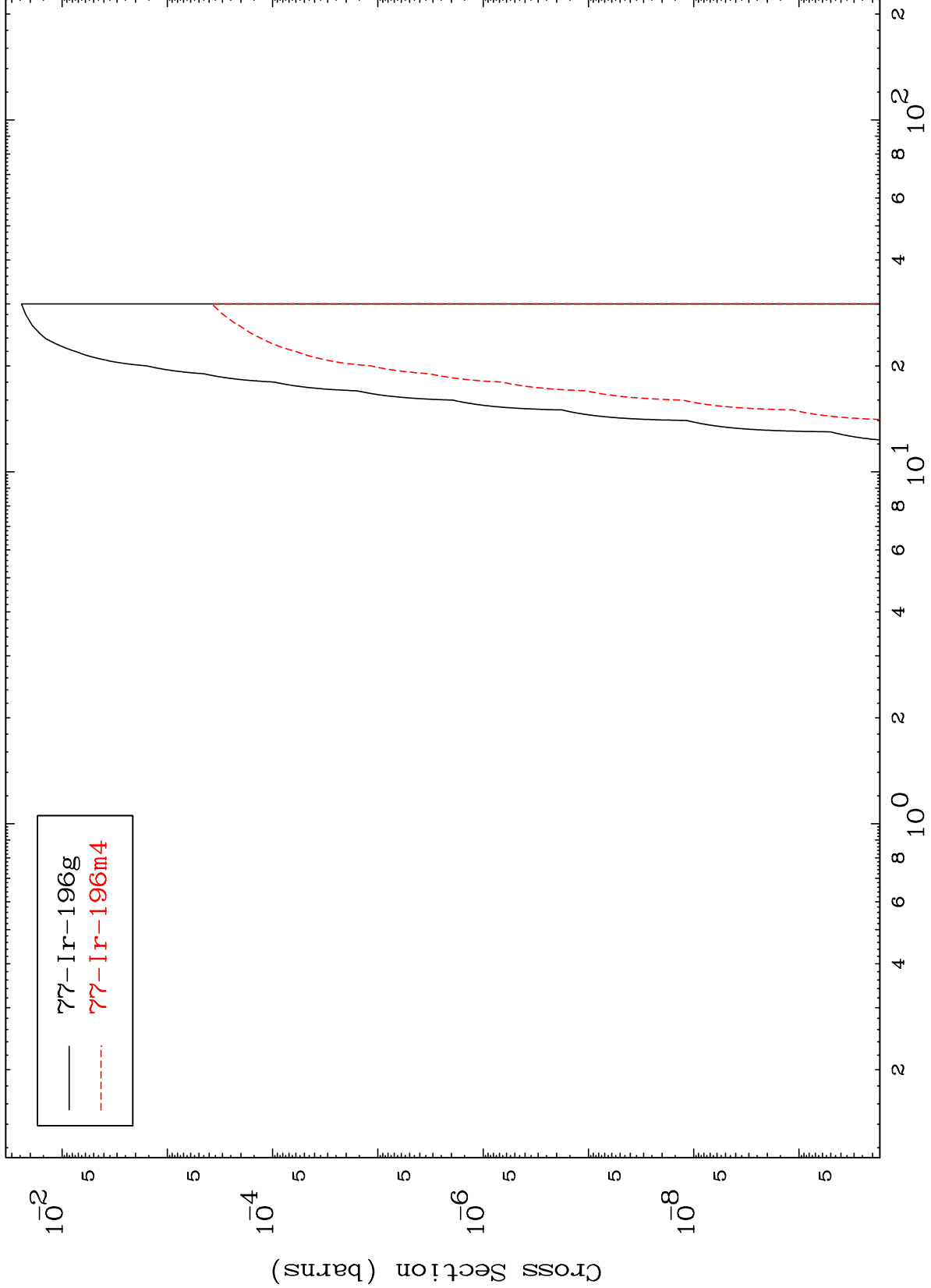
<sup>77</sup>Ir-196

MAT 7740

(n,He-3)

<sup>77</sup>Ir-196

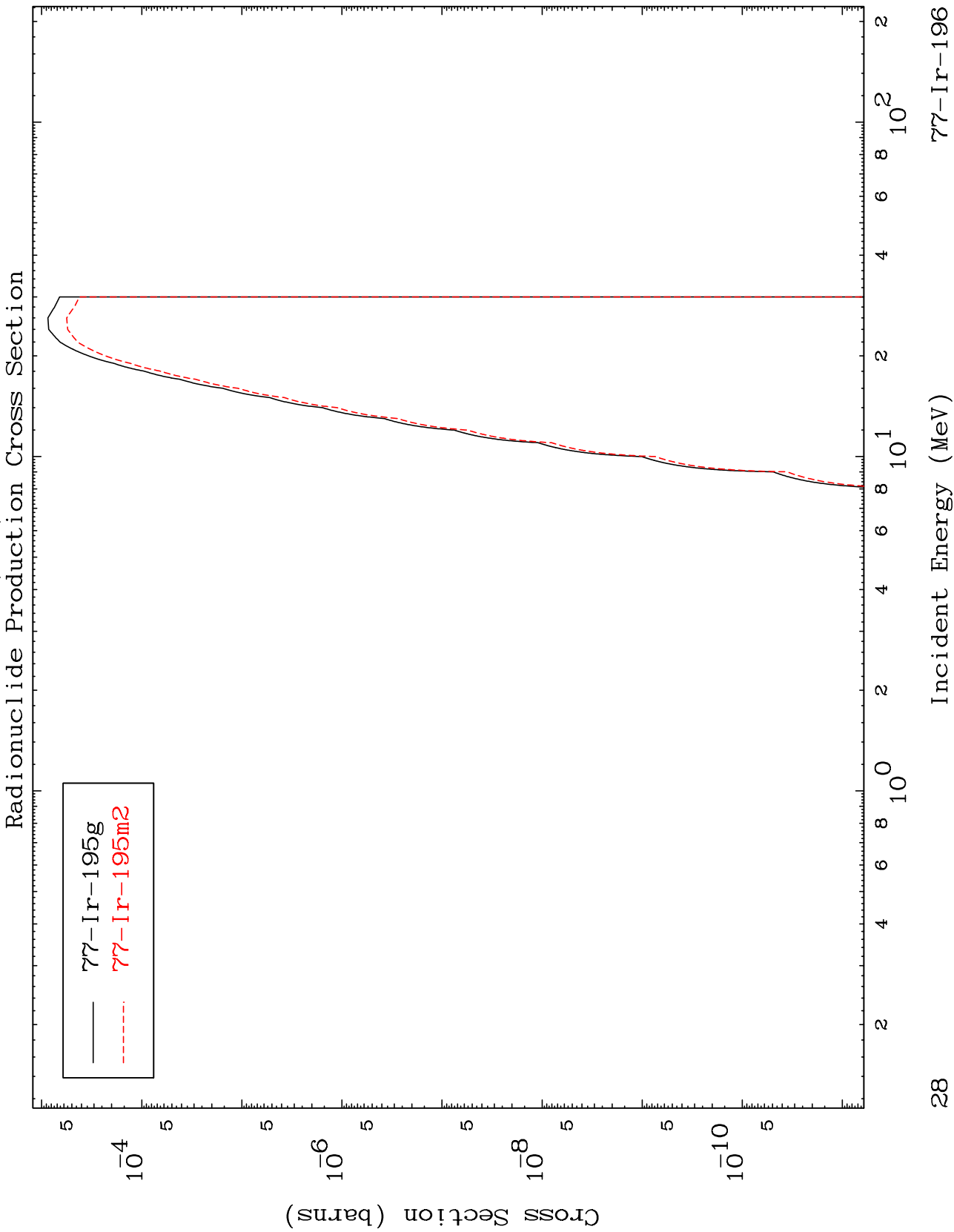
Radionuclide Production Cross Section



— <sup>77</sup>Ir-196g  
- - - <sup>77</sup>Ir-196m4

MAT 7740

77-Ir-196

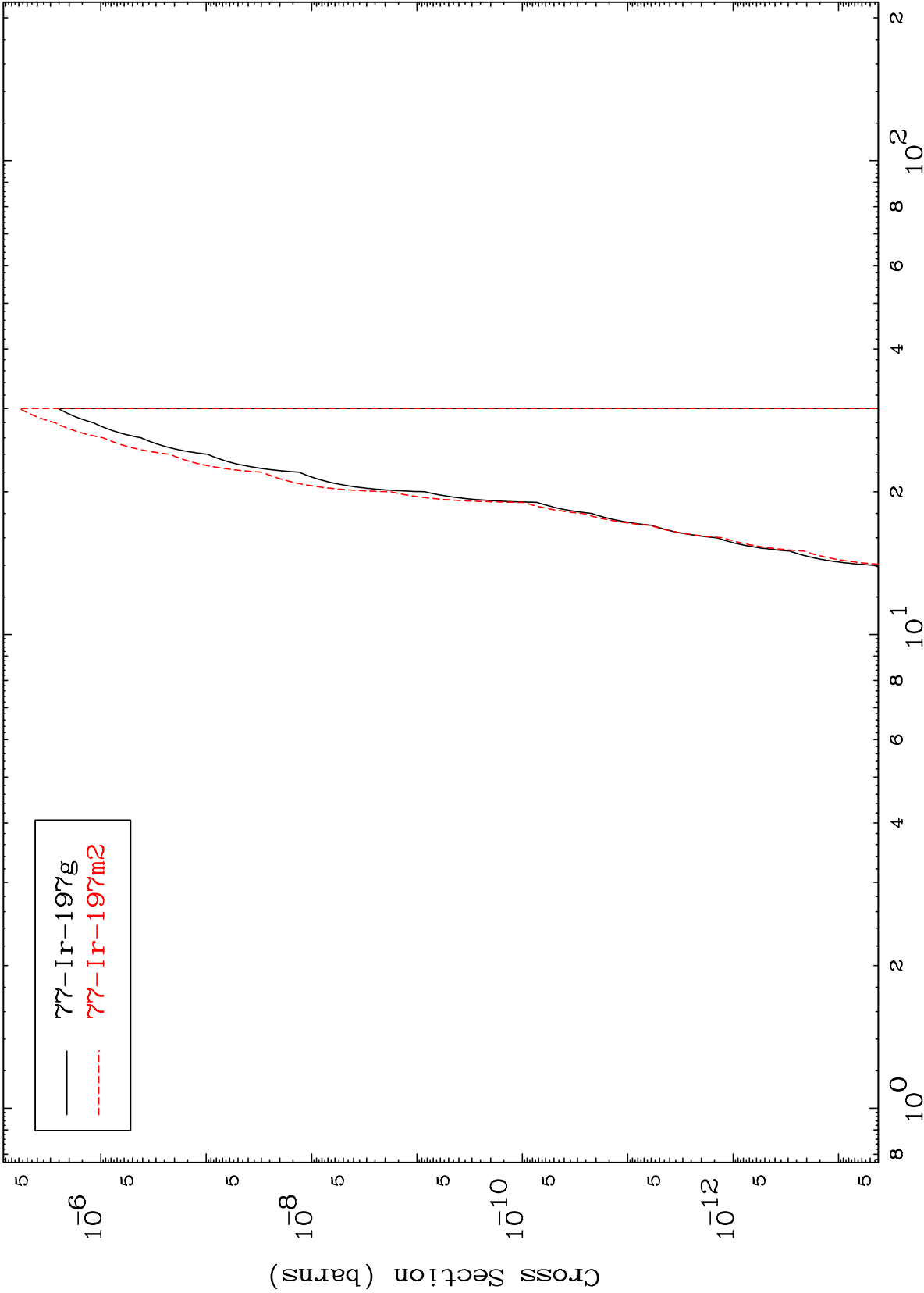


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(n,2p)

<sup>77</sup>Ir-196

Radionuclide Production Cross Section



29

Incident Energy (MeV)

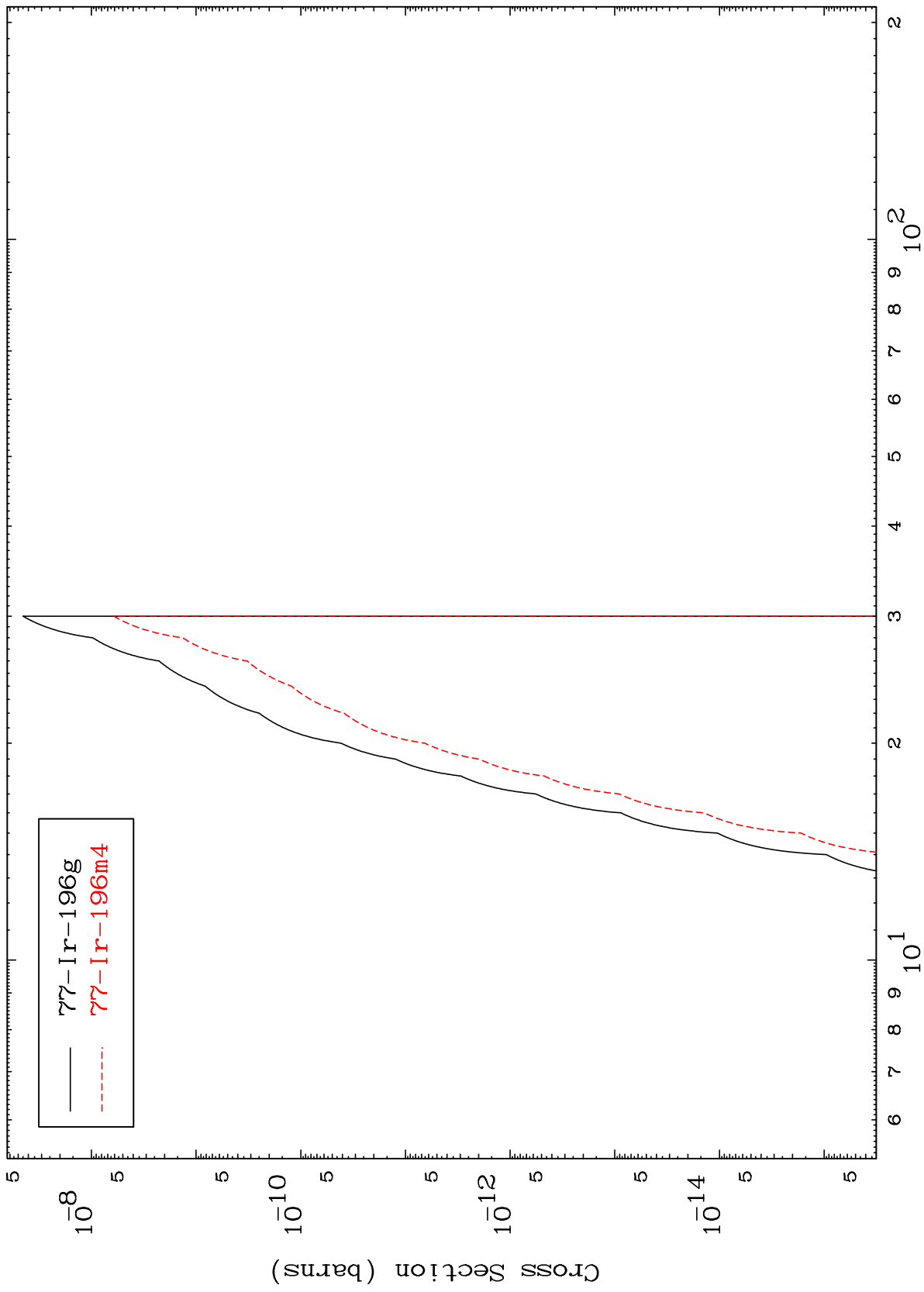
<sup>77</sup>Ir-196

MAT 7740

<sup>77</sup>Ir-196

(n,p) d

Radionuclide Production Cross Section



30

Incident Energy (MeV)

<sup>77</sup>Ir-196

MAT 7740

(n,p) t

<sup>77</sup>Ir-196

Radionuclide Production Cross Section

