

Program EVALPLOT
(Version 2021-1)

by

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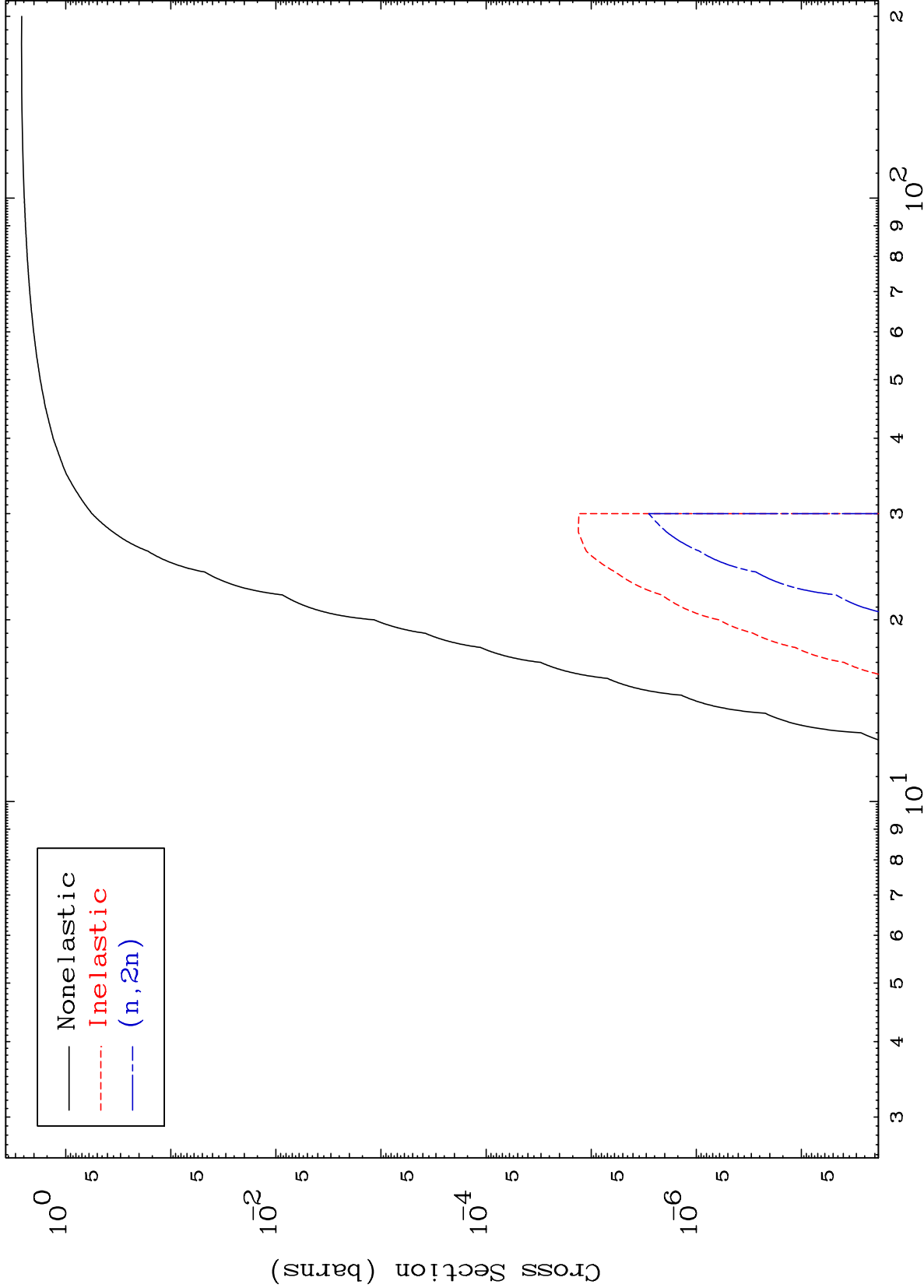
Press Mouse Button to Start

MAT 9952

He-3 Major

101-Md-251

0 Kelvin Cross Sections



1

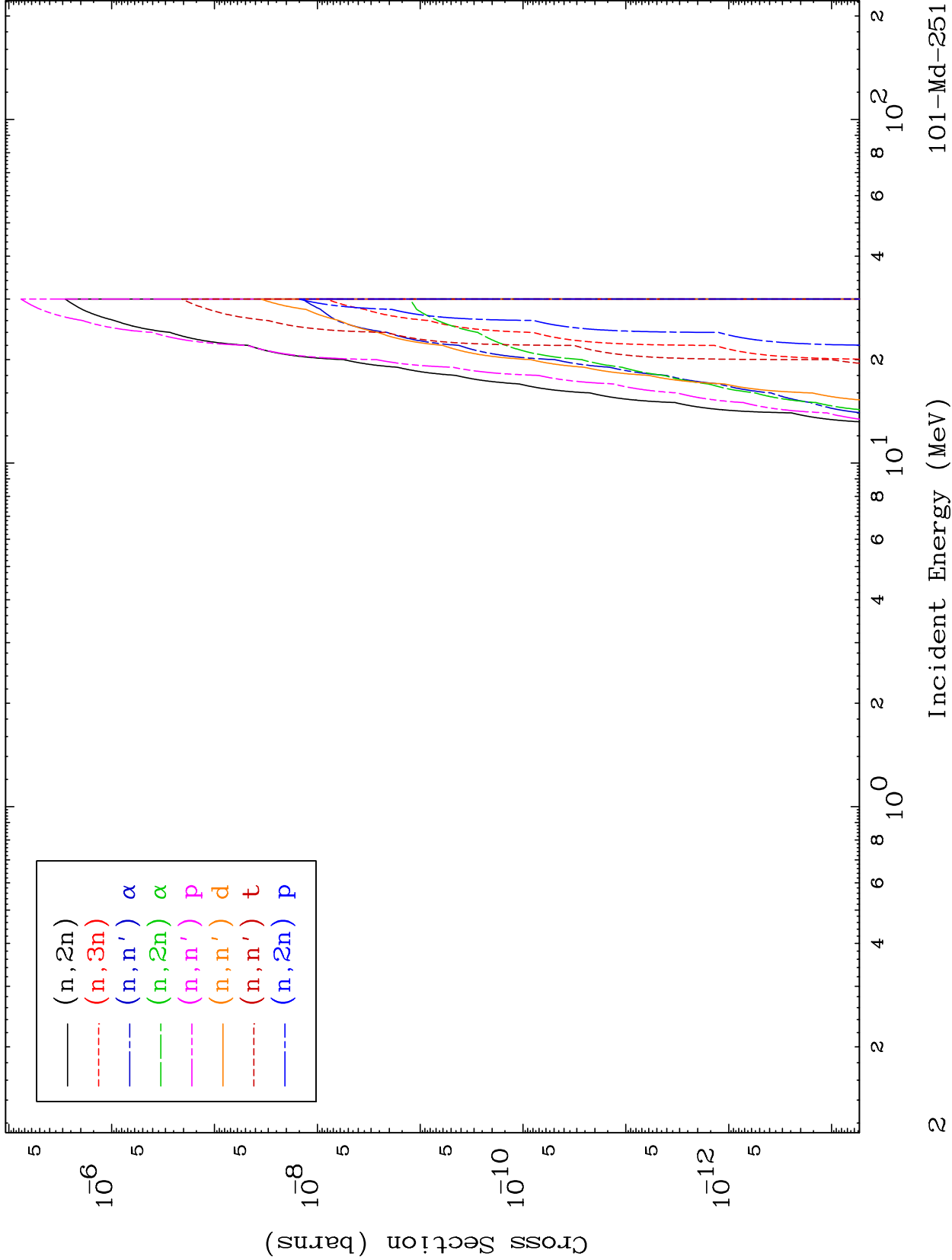
Incident Energy (MeV)

101-Md-251

MAT 9952

He-3 Neutron Absorption
0 Kelvin Cross Sections

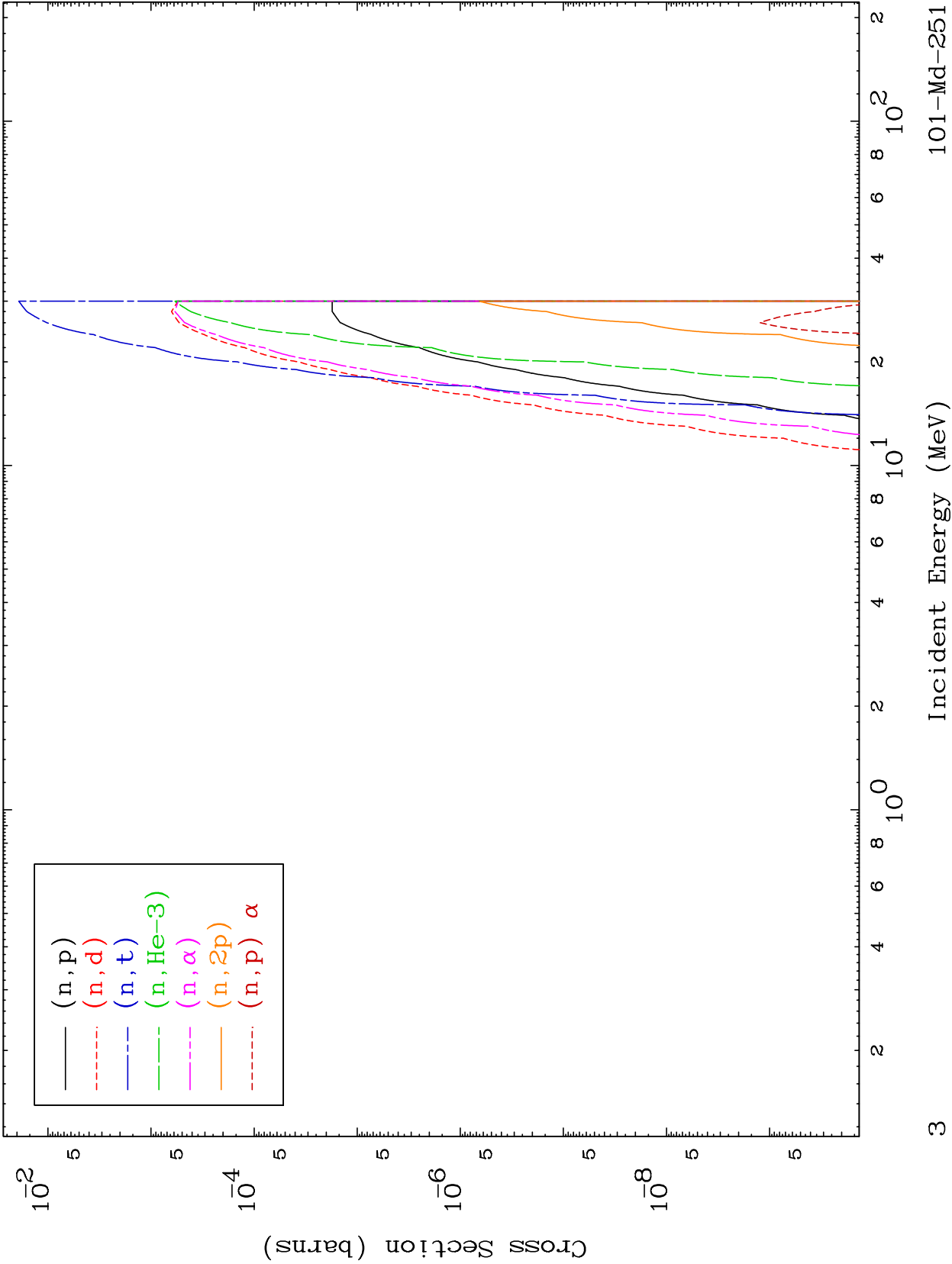
101-Md-251



MAT 9952

He-3 Neutron Absorption
0 Kelvin Cross Sections

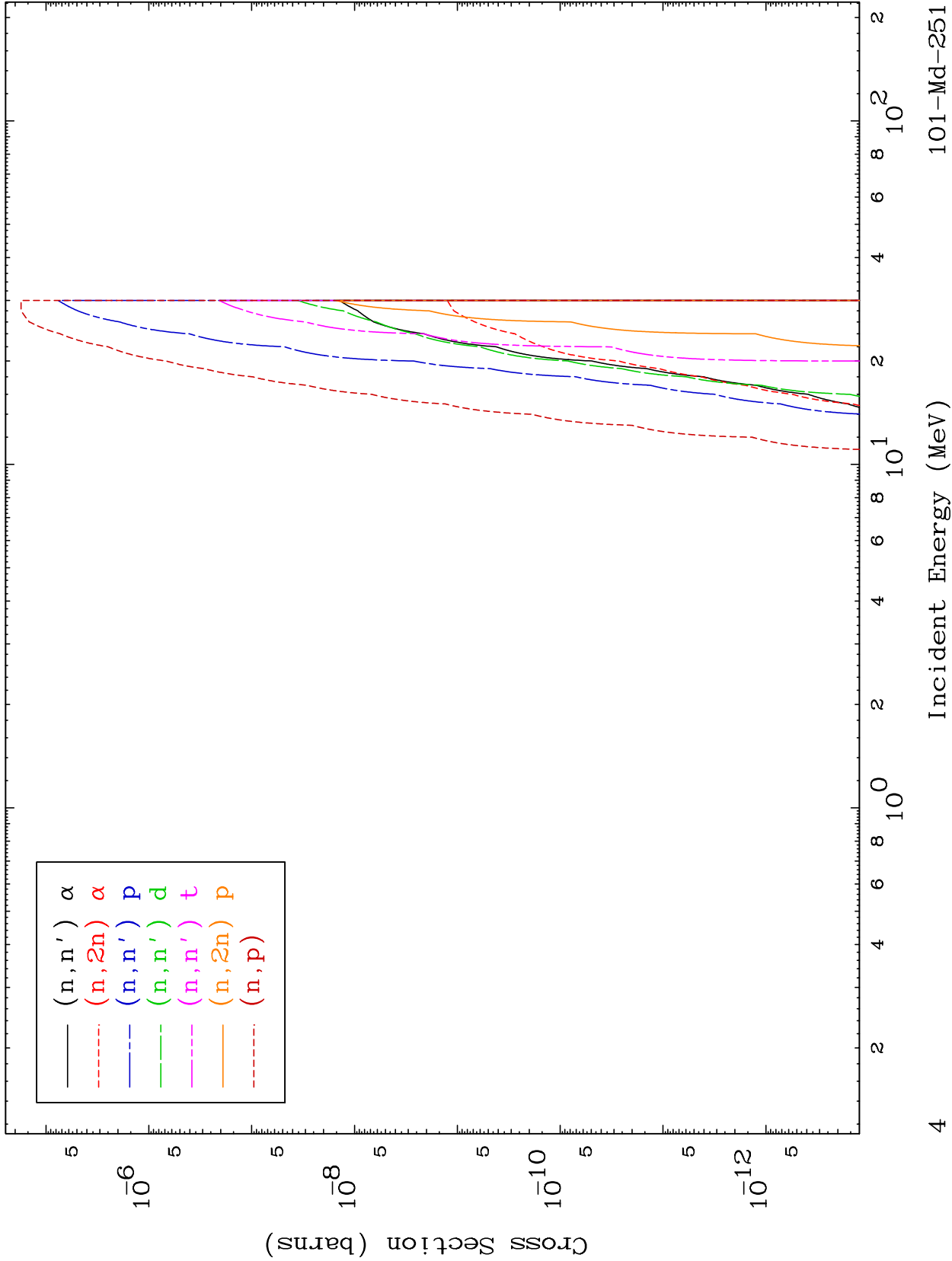
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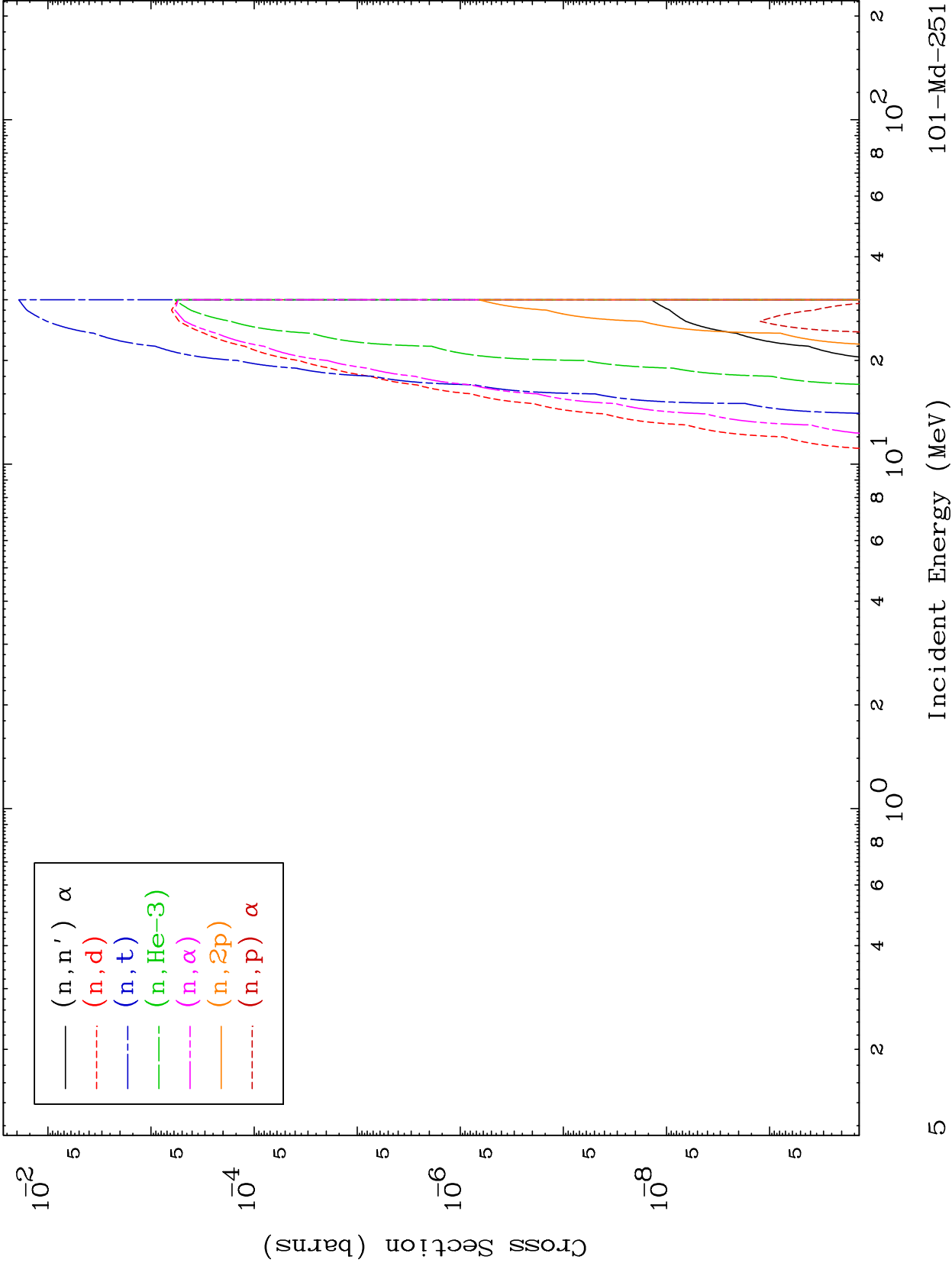


MAT 9952

He-3 Charged Particle
0 Kelvin Cross Sections

101-Md-251



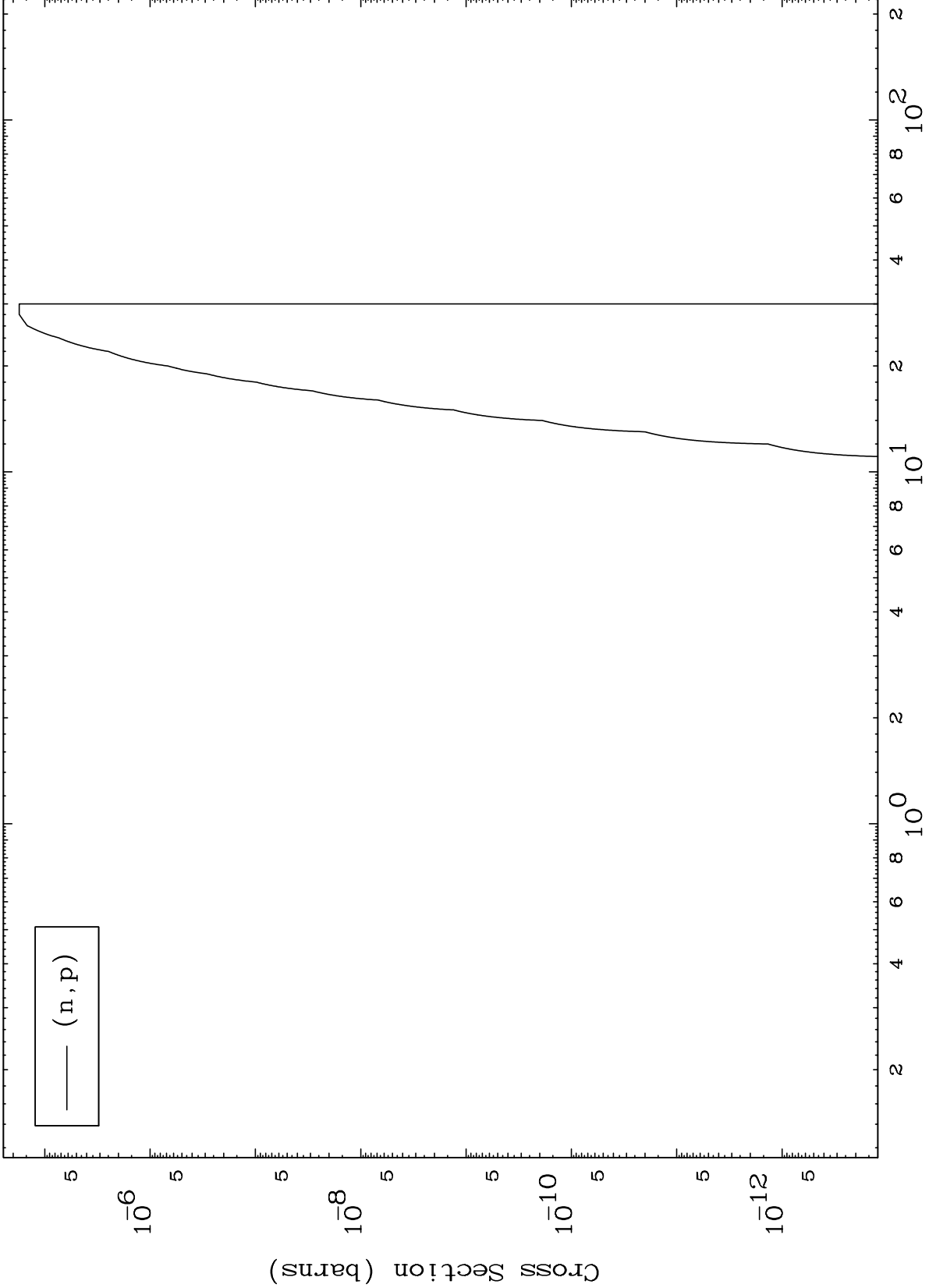


MAT 9952

(He-3,p) Levels

101-Md-251

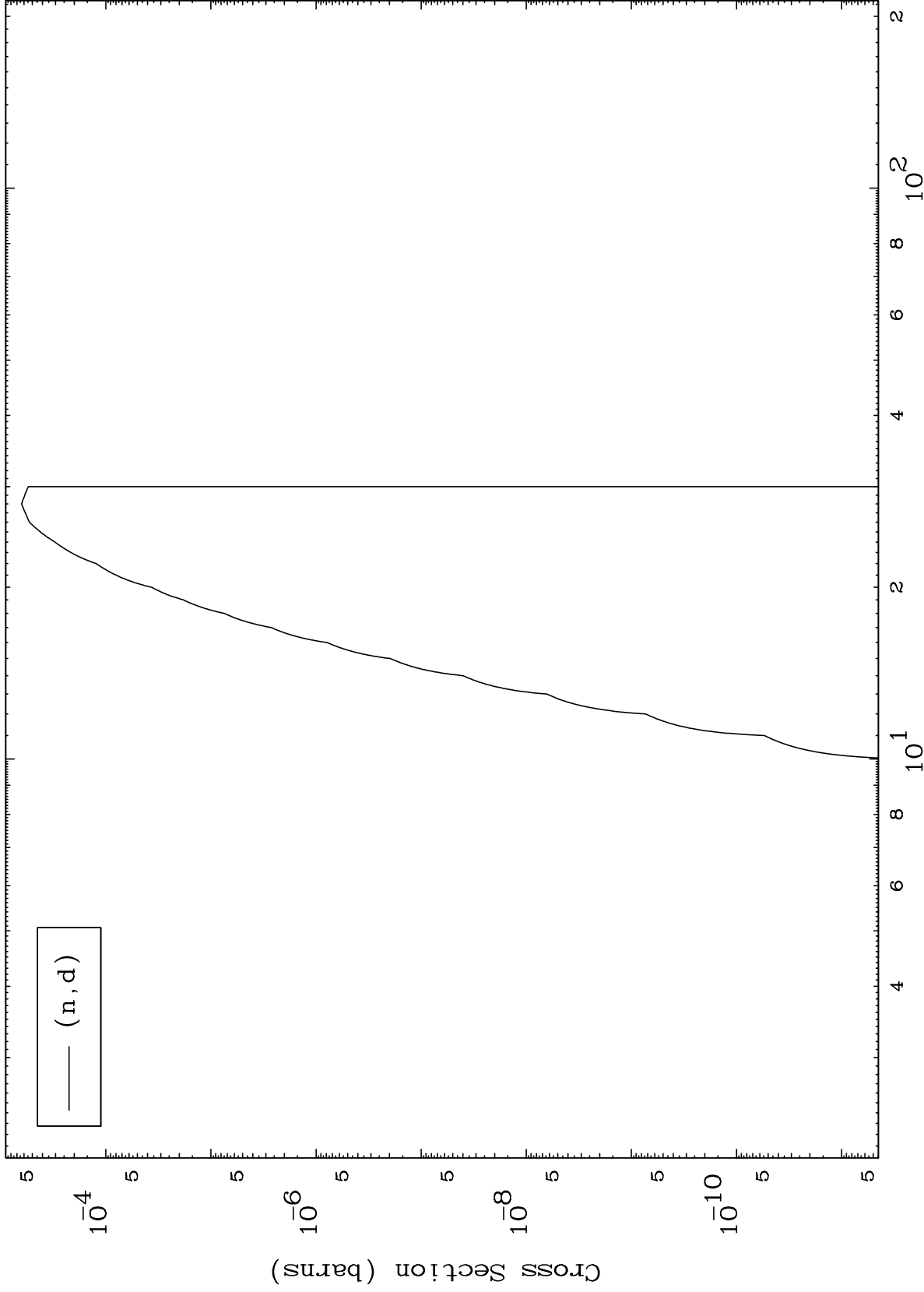
0 Kelvin Cross Sections



MAT 9952

(He-3,d) Levels
0 Kelvin Cross Sections

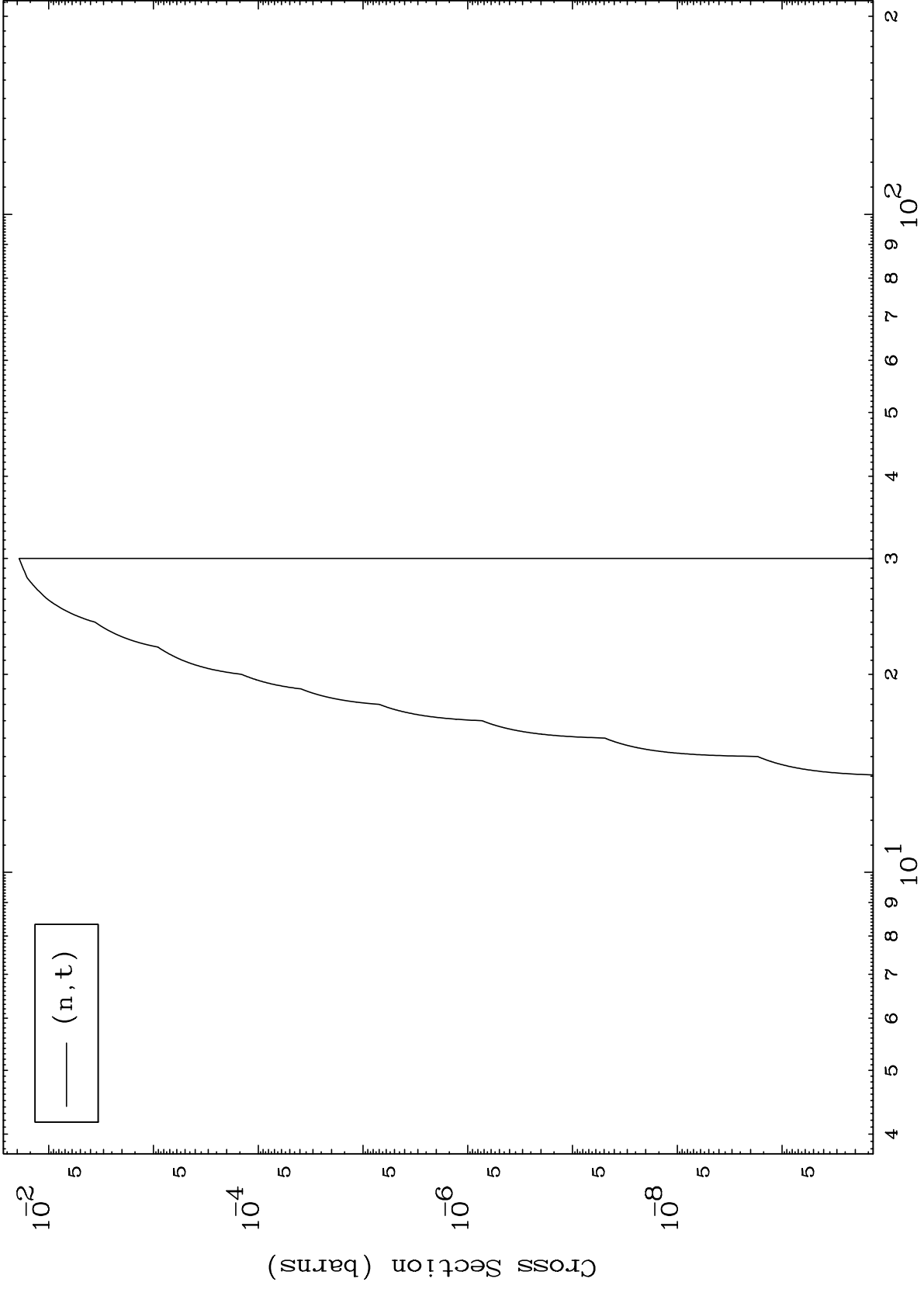
101-Md-251



MAT 9952

101-Md-251

(He-3,t) Levels
0 Kelvin Cross Sections



101-Md-251

Incident Energy (MeV)

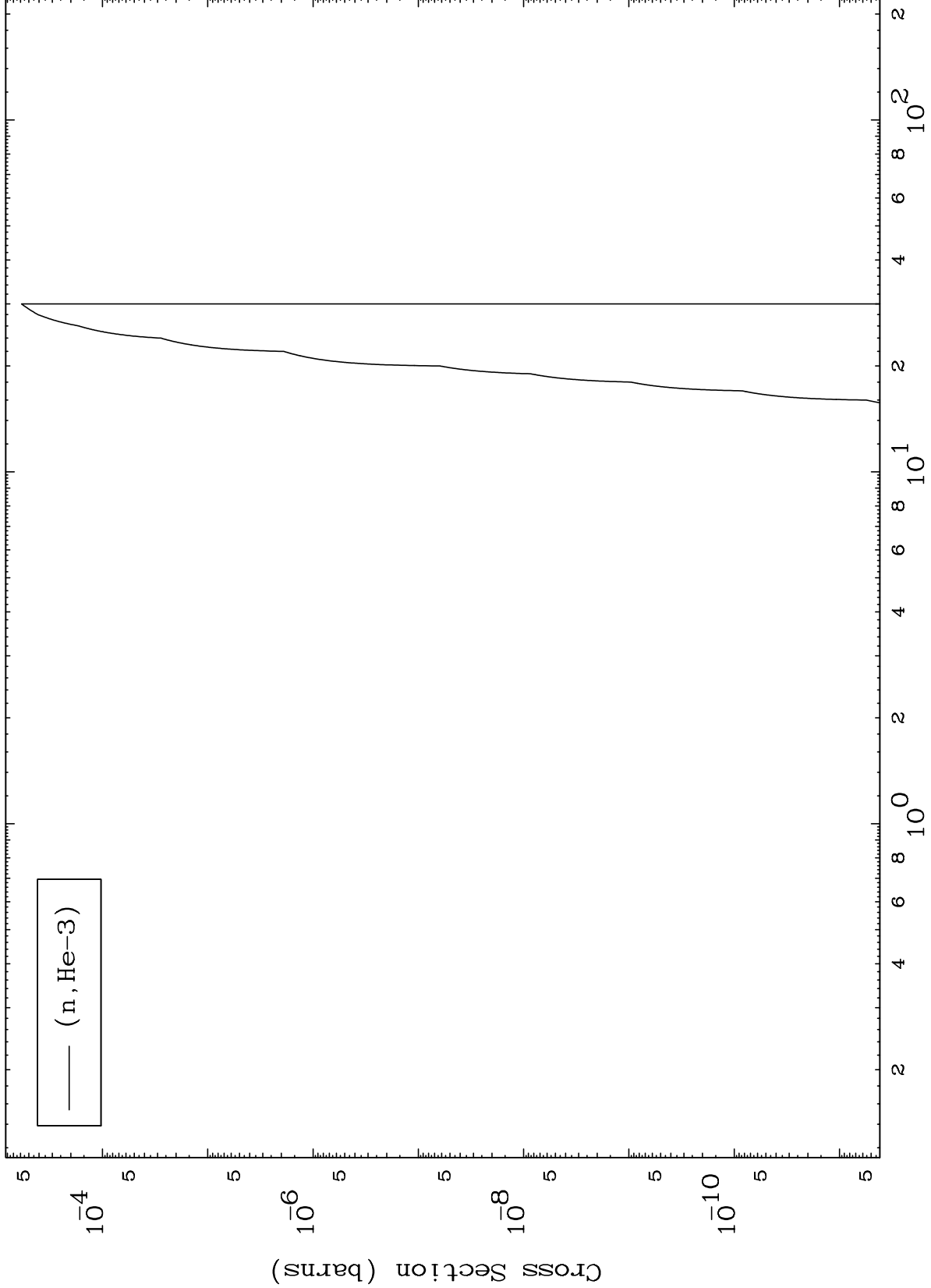
8

MAT 9952

(He-3, He3) Levels

101-Md-251

0 Kelvin Cross Sections

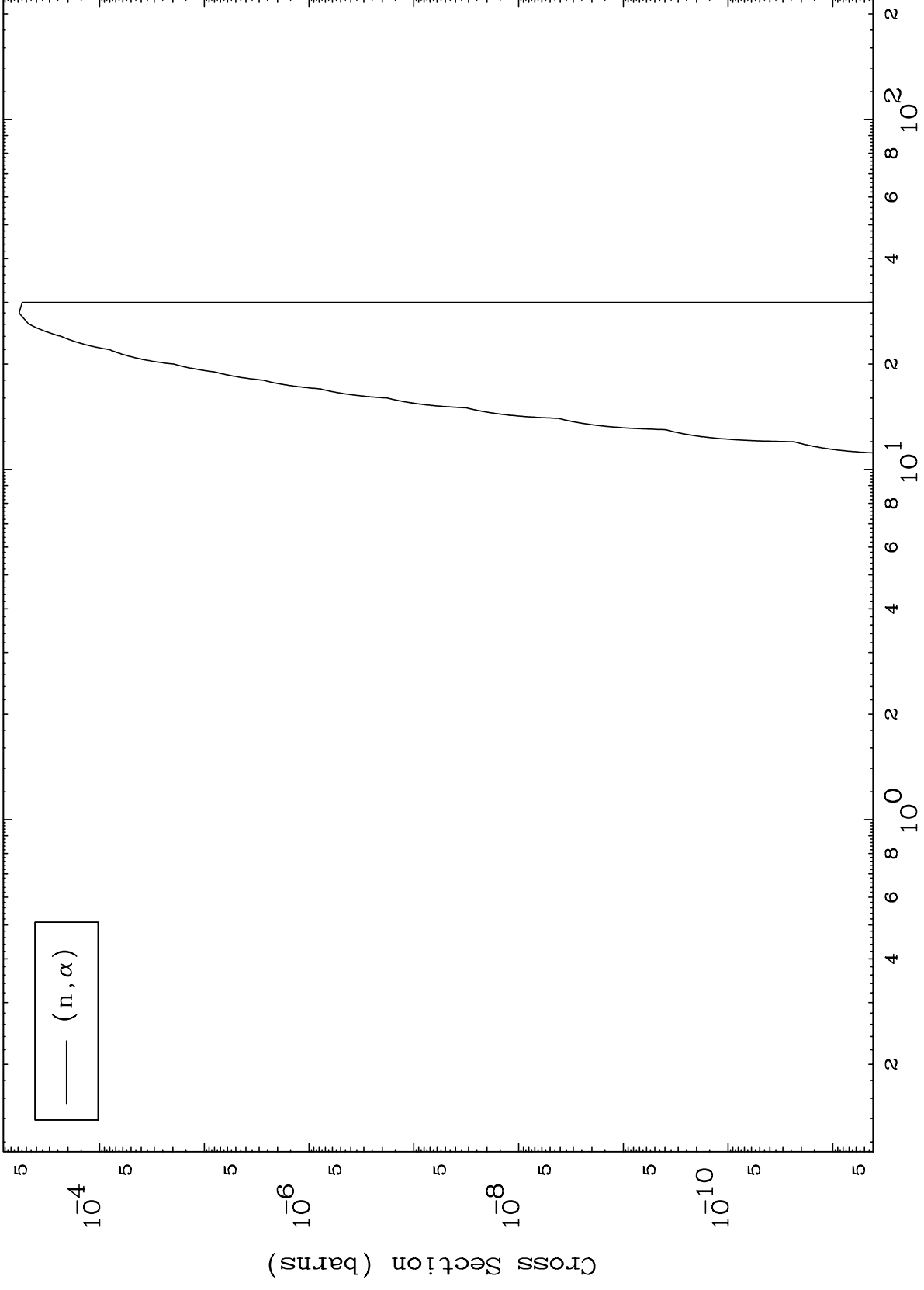


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(He-3, α) Levels

101-Md-251

0 Kelvin Cross Sections



10

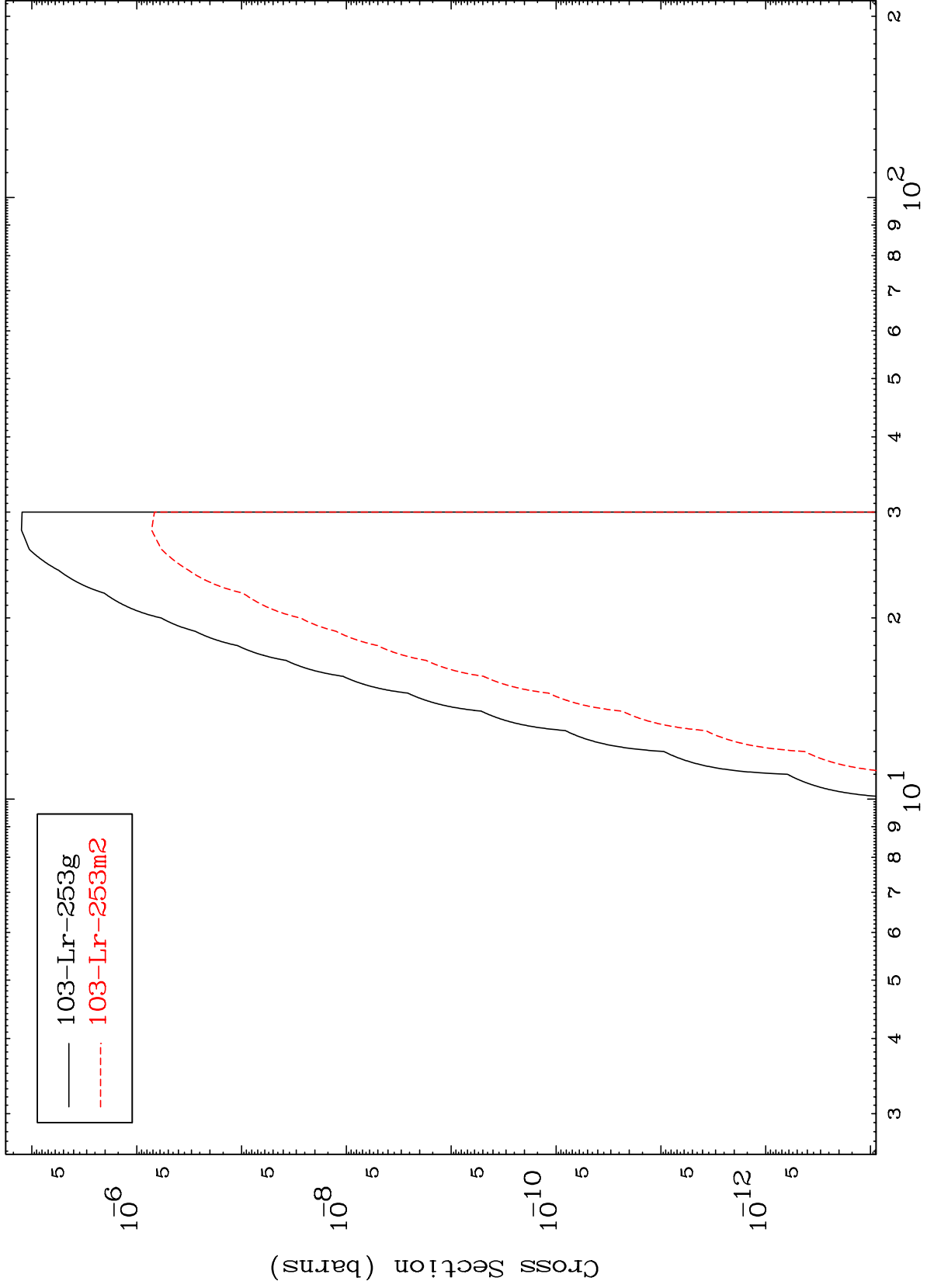
Incident Energy (MeV)

101-Md-251

MAT 9952

101-Md-251

Radionuclide Production Cross Section



101-Md-251

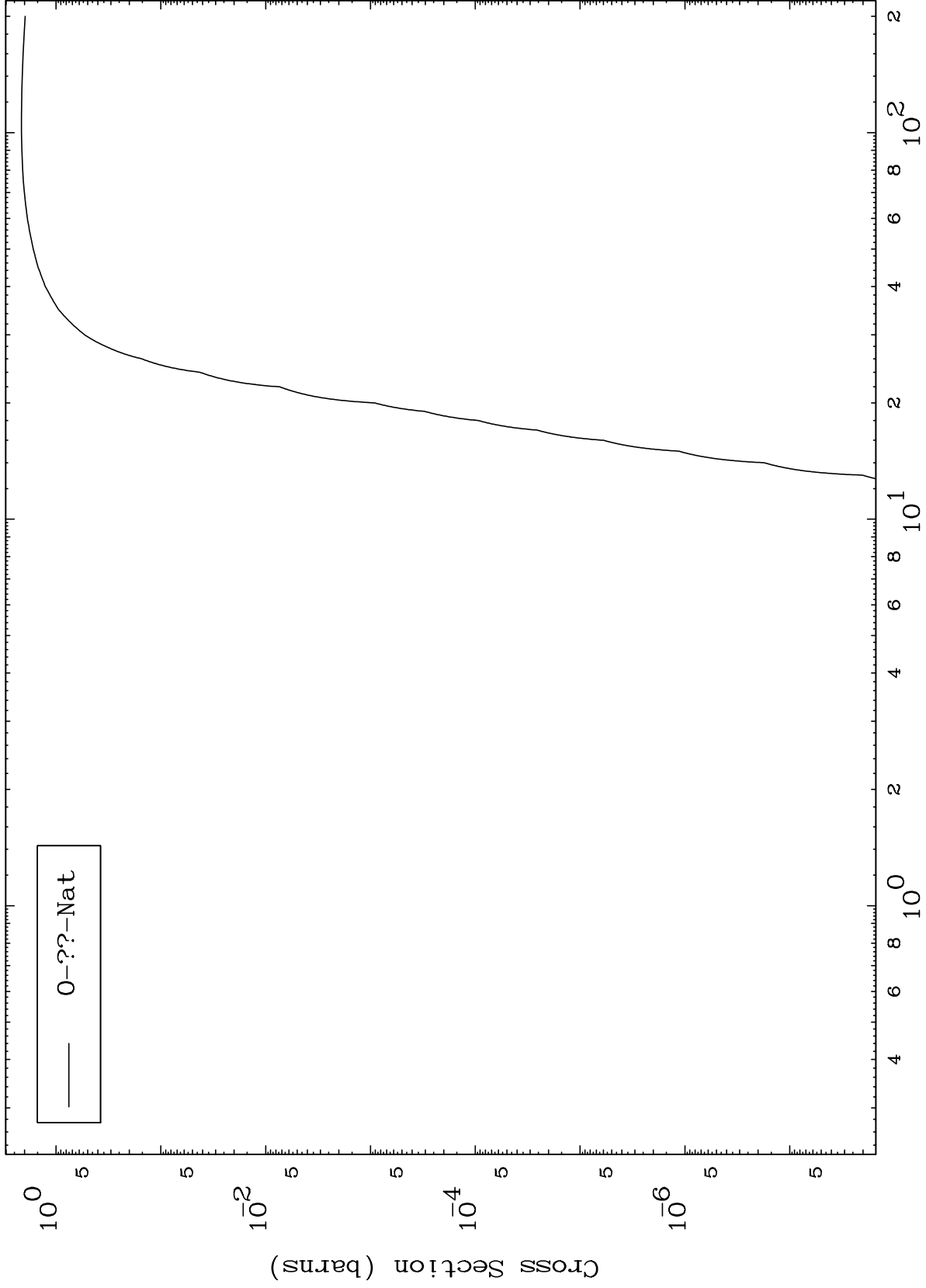
Incident Energy (MeV)

MAT 9952

Fission

101-Md-251

Radionuclide Production Cross Section



12

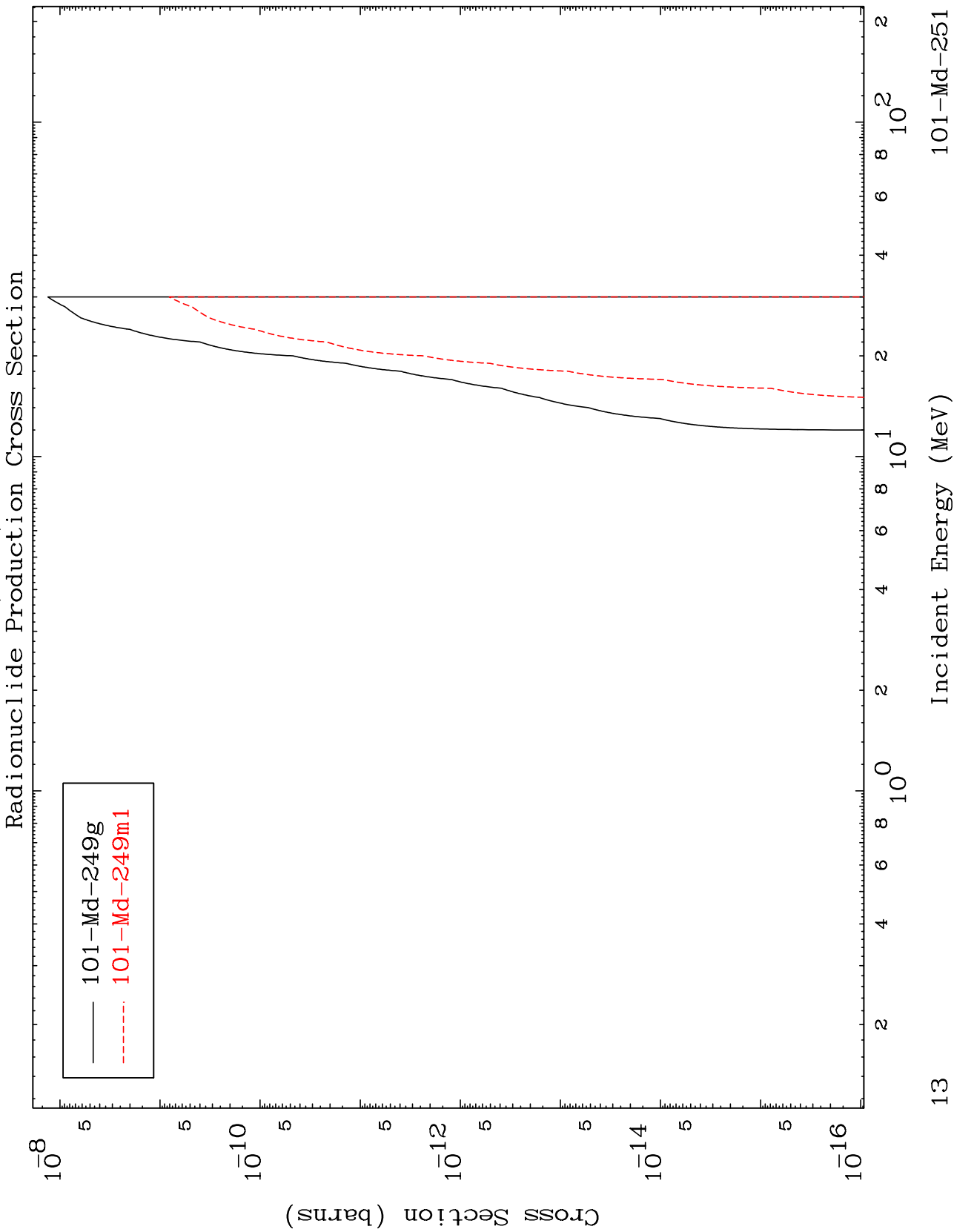
Incident Energy (MeV)

101-Md-251

MAT 9952

(n,n') α

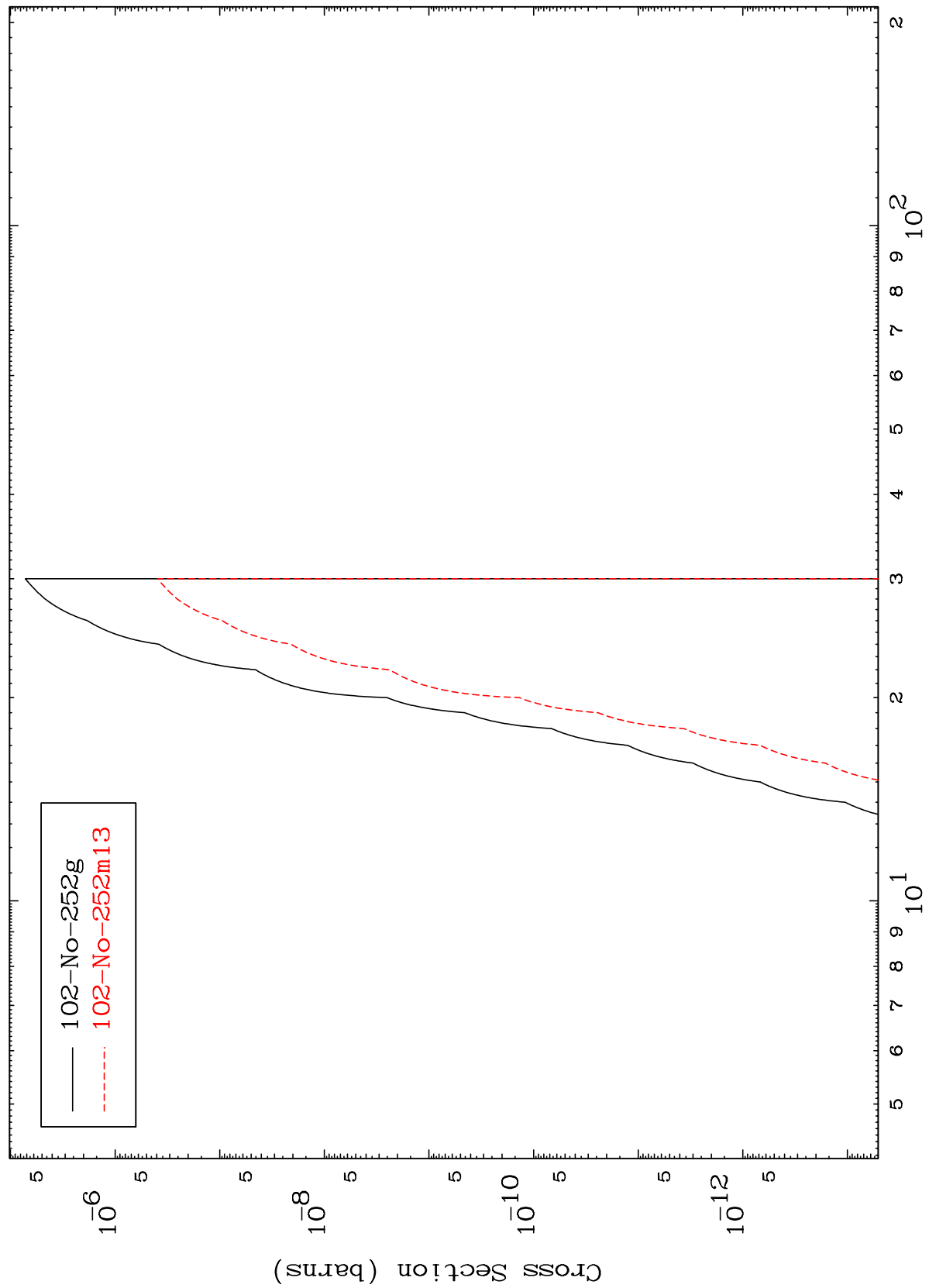
101-Md-251



MAT 9952

101-Md-251

(n,n') p
Radionuclide Production Cross Section



14

101-Md-251

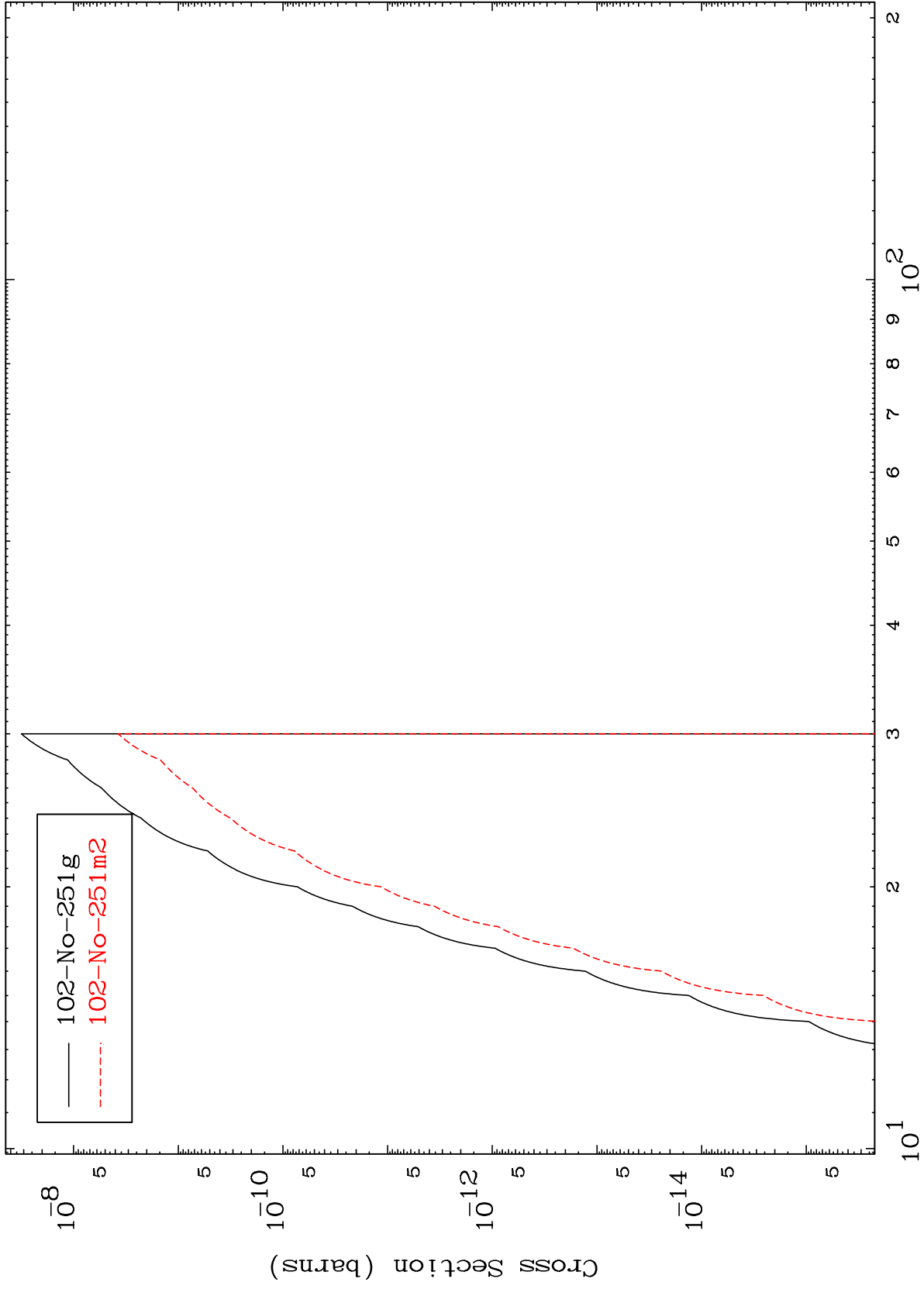
Incident Energy (MeV)

MAT 9952

(n,n') d

101-Md-251

Radionuclide Production Cross Section



Incident Energy (MeV)

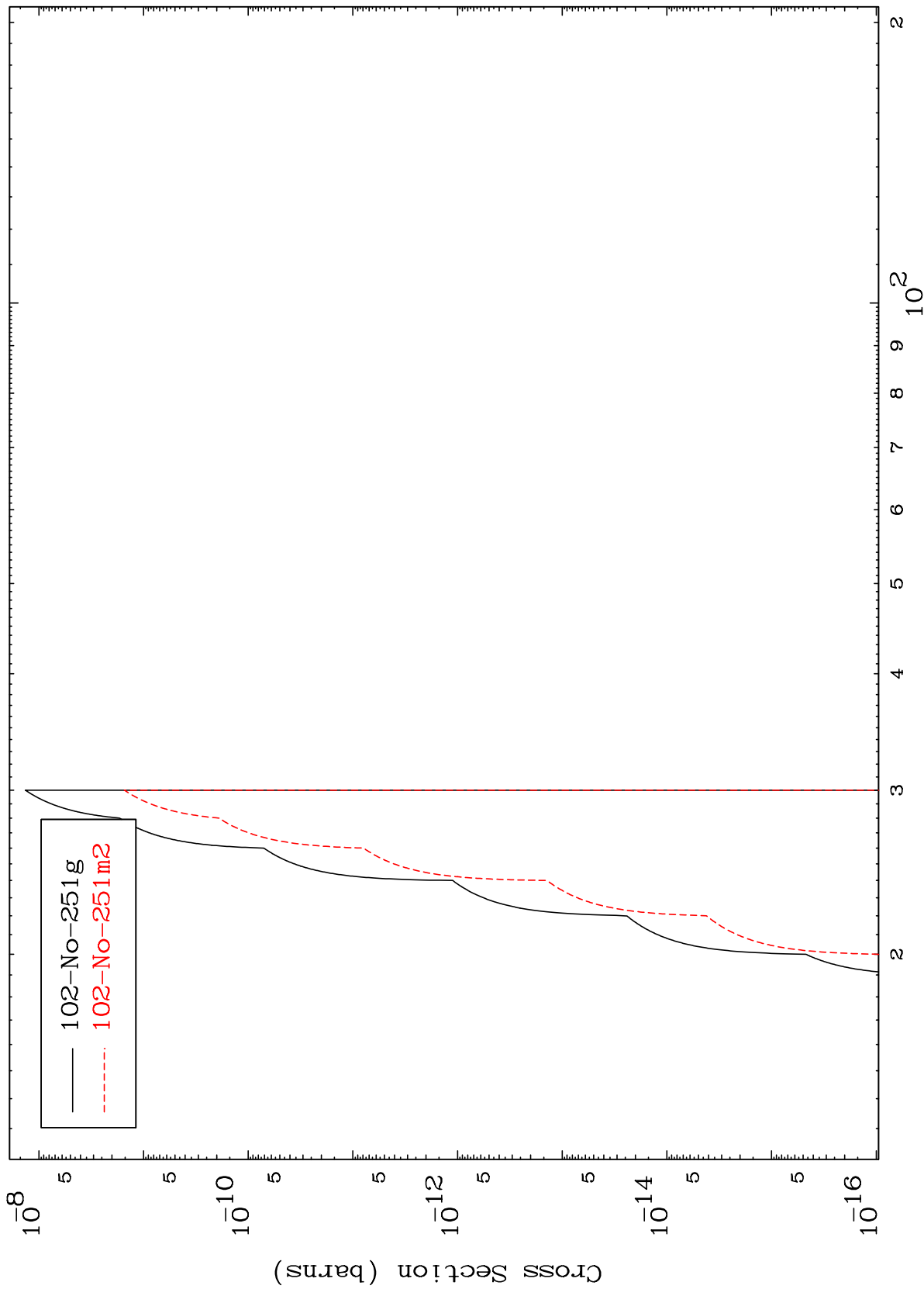
101-Md-251

MAT 9952

(n,2n) p

101-Md-251

Radionuclide Production Cross Section



16

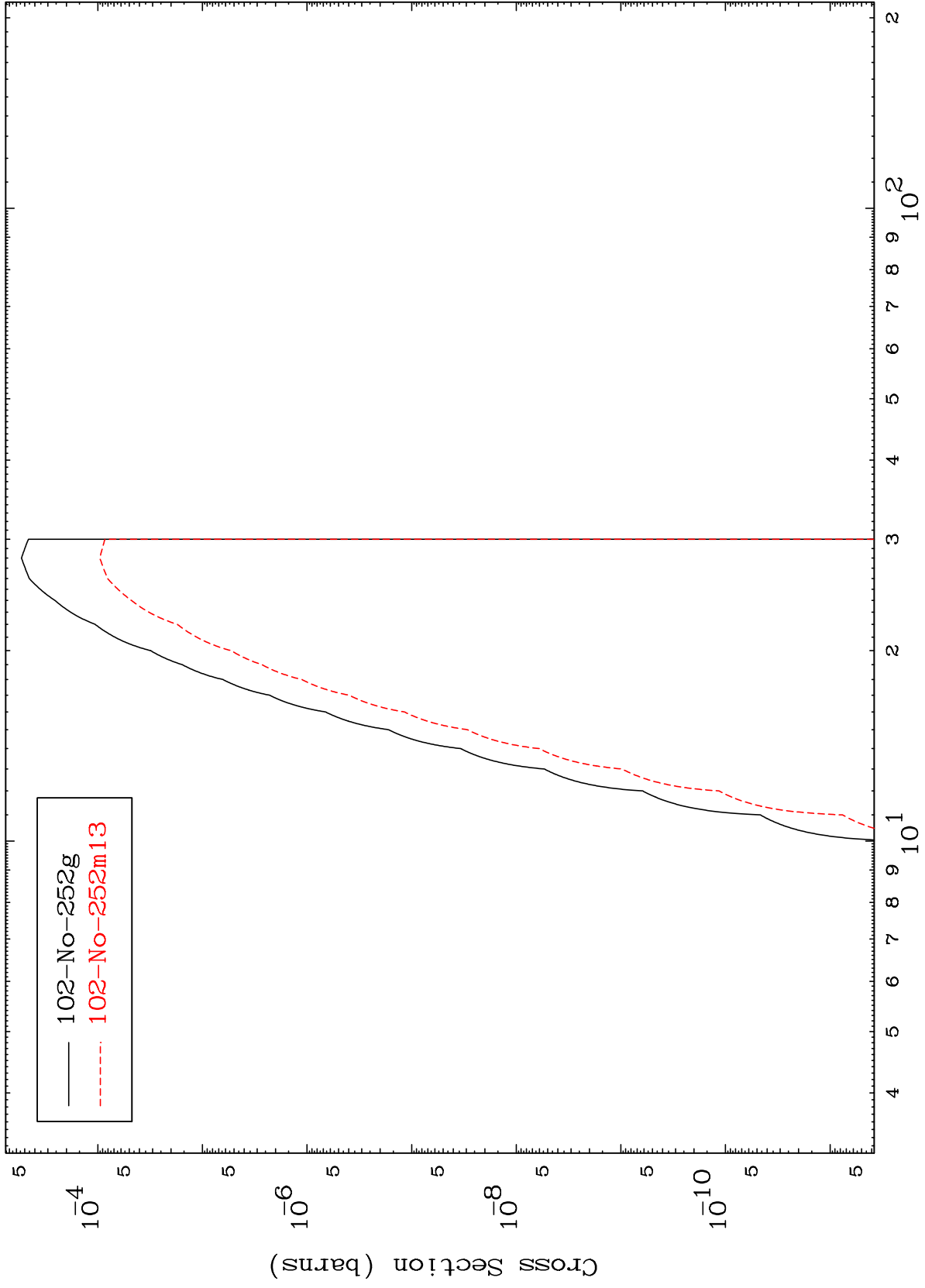
Incident Energy (MeV)

101-Md-251

MAT 9952

101-Md-251

(n,d)
Radionuclide Production Cross Section



101-Md-251

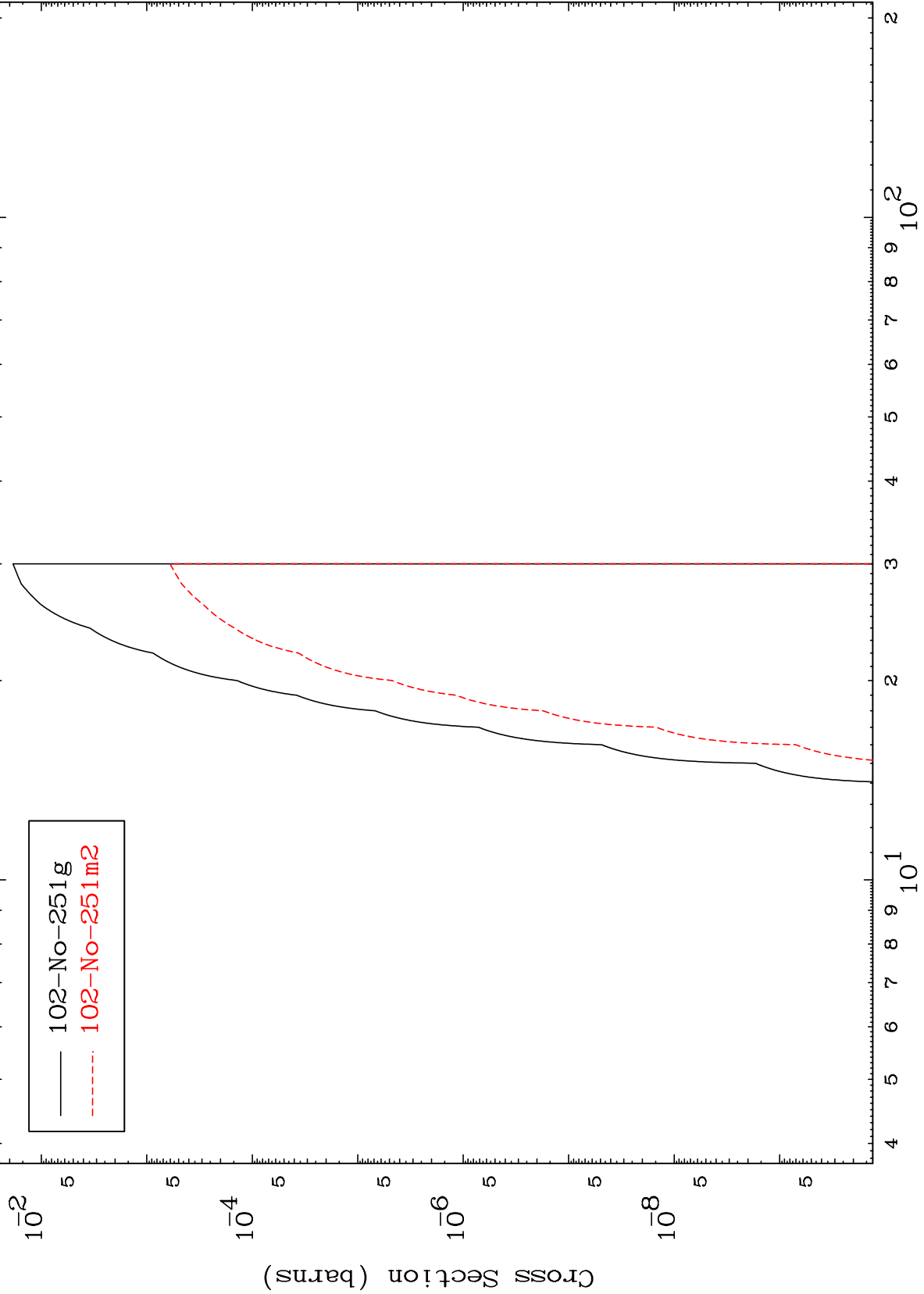
Incident Energy (MeV)

17

MAT 9952

101-Md-251

(n, t)
Radionuclide Production Cross Section



— 102-No-251g
- - - 102-No-251m2

18

Incident Energy (MeV)

101-Md-251