

Program EVALPLOT
(Version 2021-1)

by

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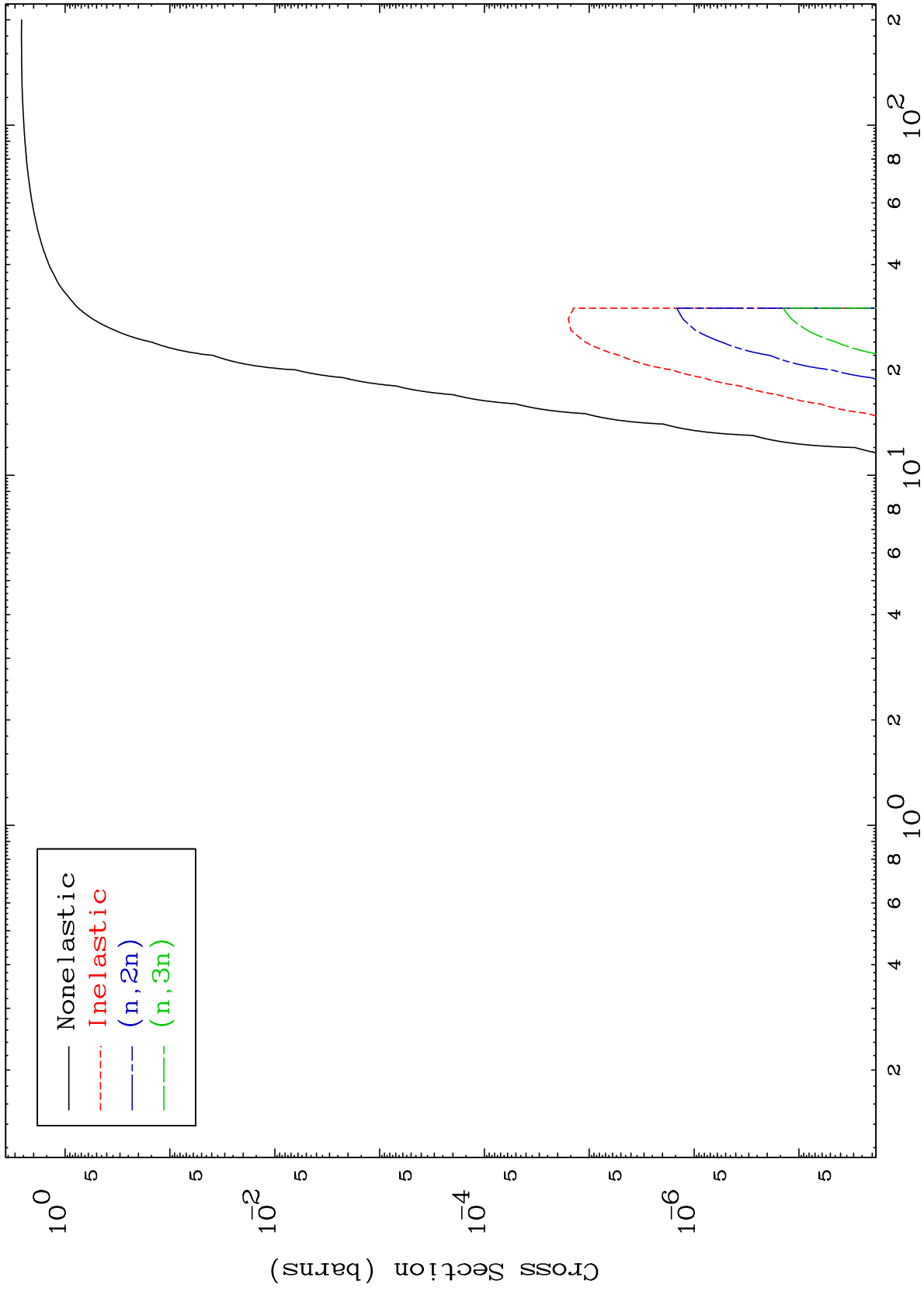
Press Mouse Button to Start

MAT 9346

He-3 Major

93-Np-237

0 Kelvin Cross Sections



1

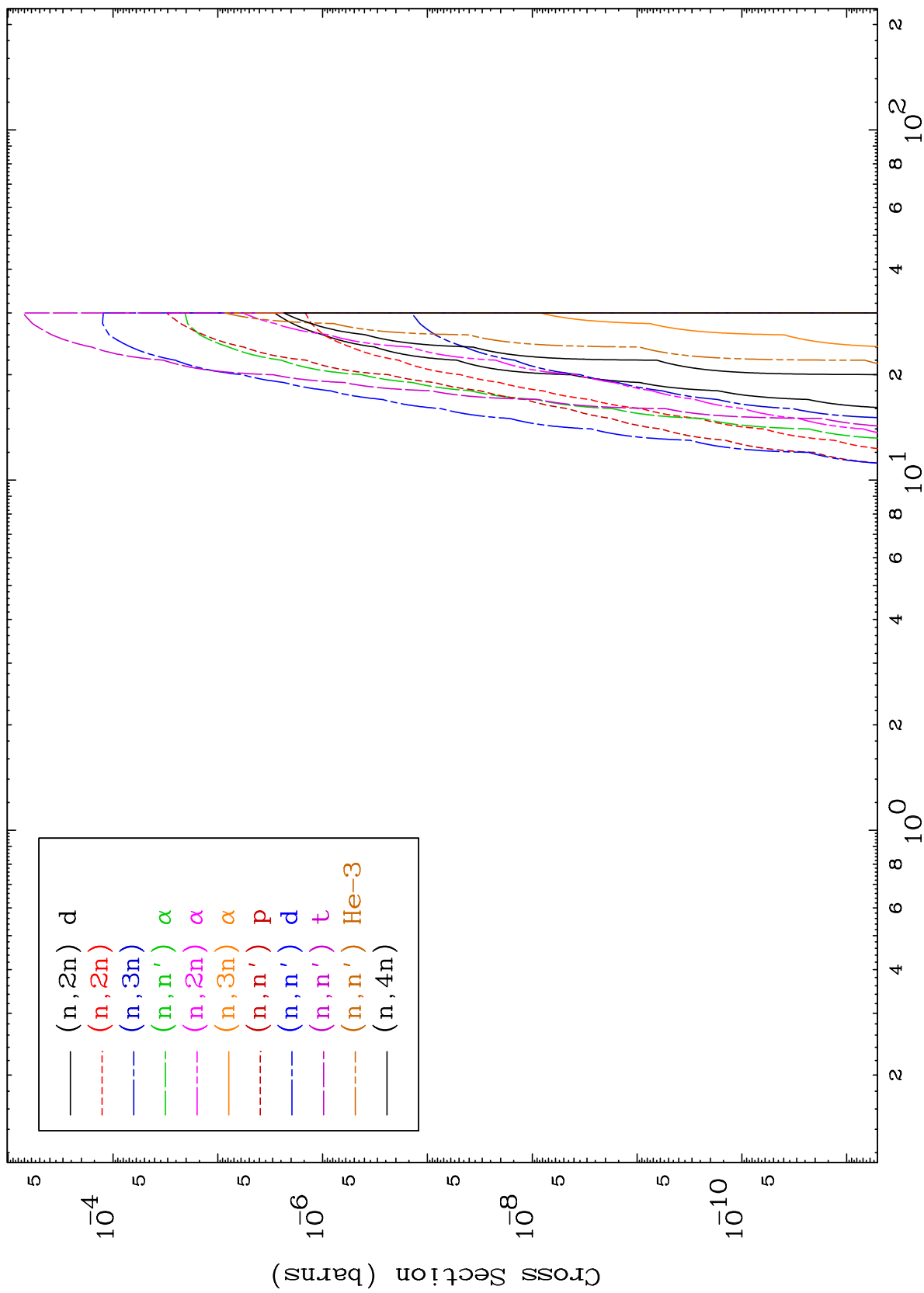
Incident Energy (MeV)

93-Np-237

MAT 9346

He-3 Neutron Absorption
0 Kelvin Cross Sections

93-Np-237



2

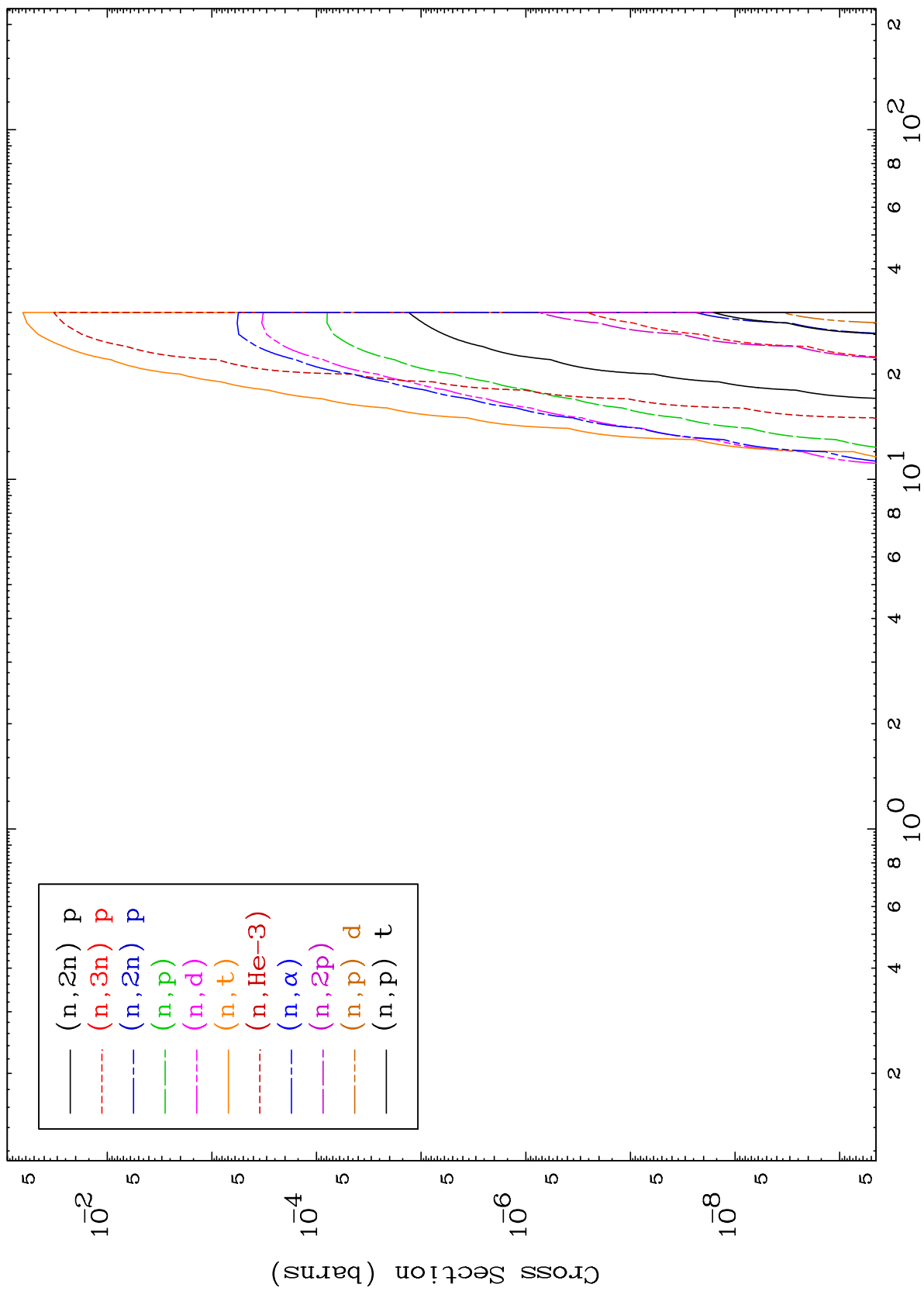
Incident Energy (MeV)

93-Np-237

MAT 9346

He-3 Neutron Absorption
0 Kelvin Cross Sections

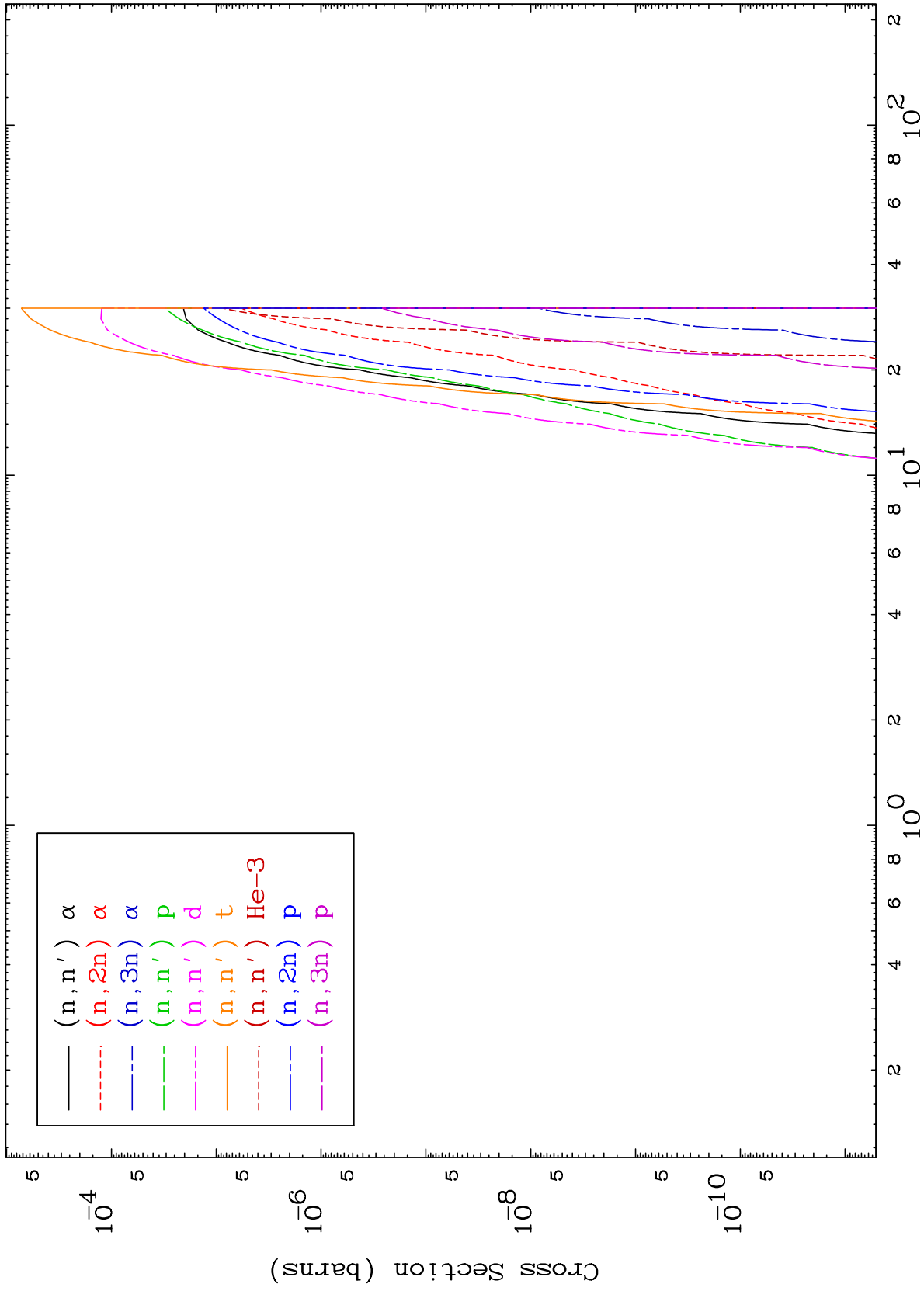
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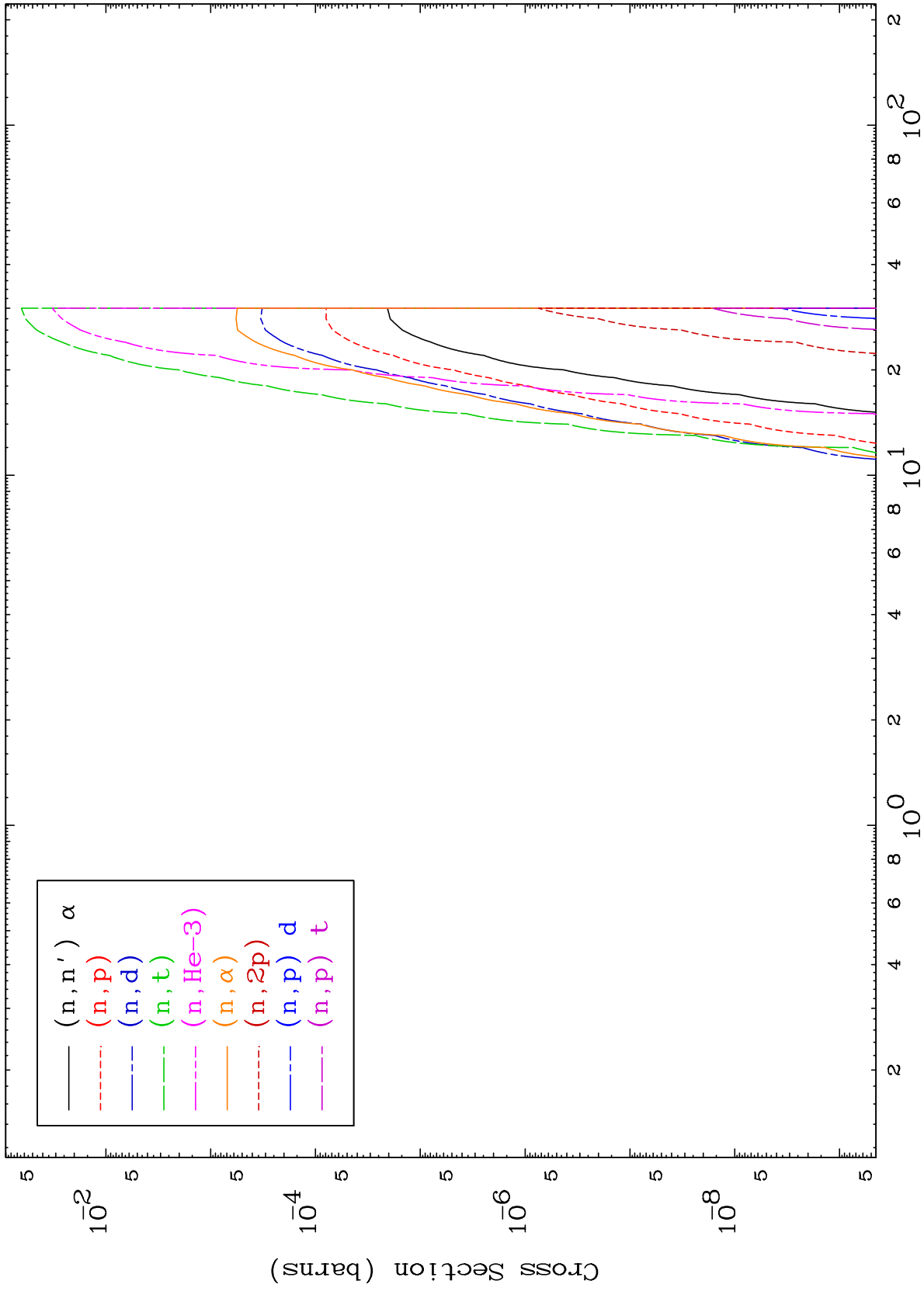


3

Incident Energy (MeV)

93-Np-237

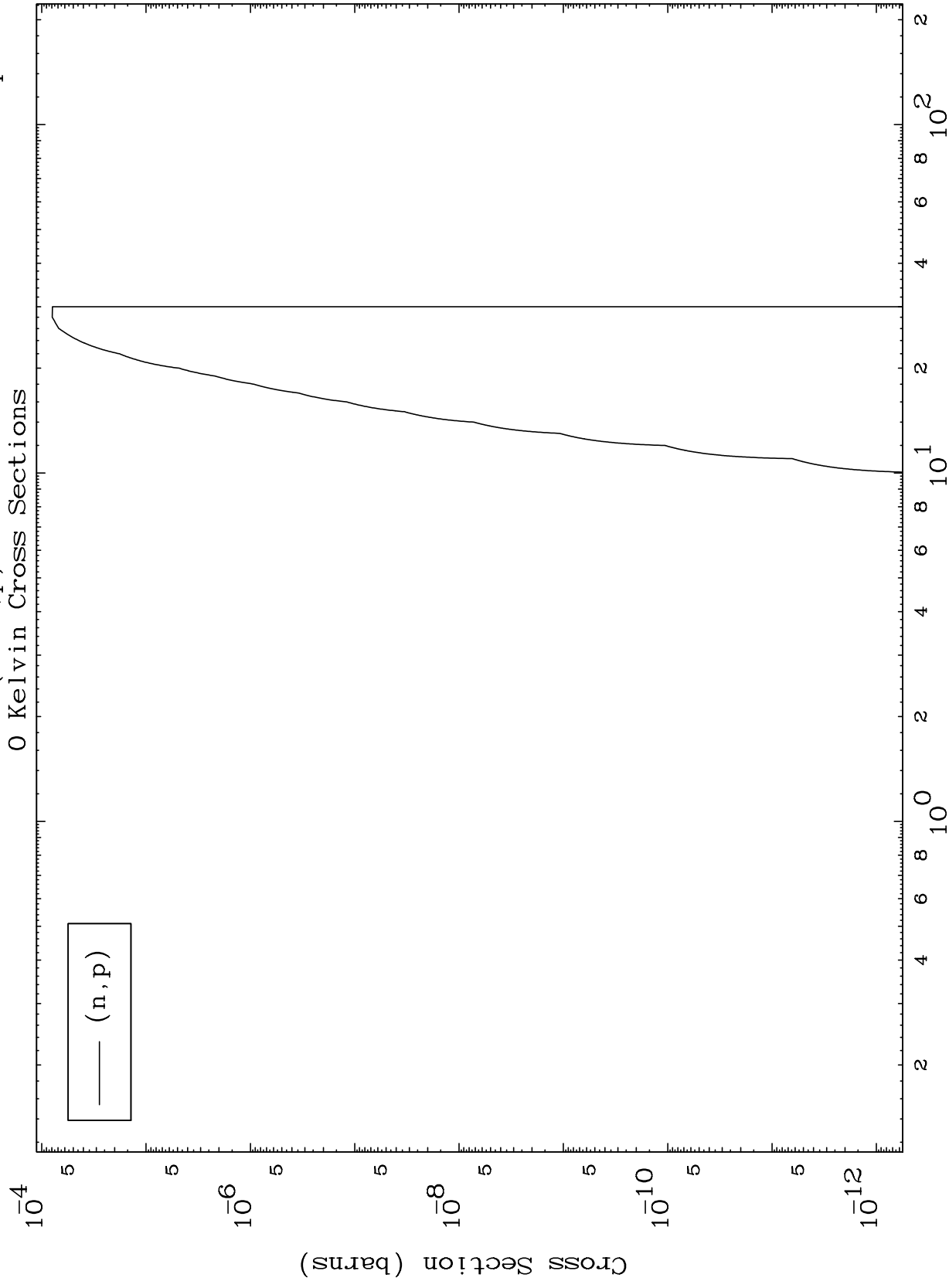




MAT 9346

(He-3,p) Levels
0 Kelvin Cross Sections

93-Np-237



6

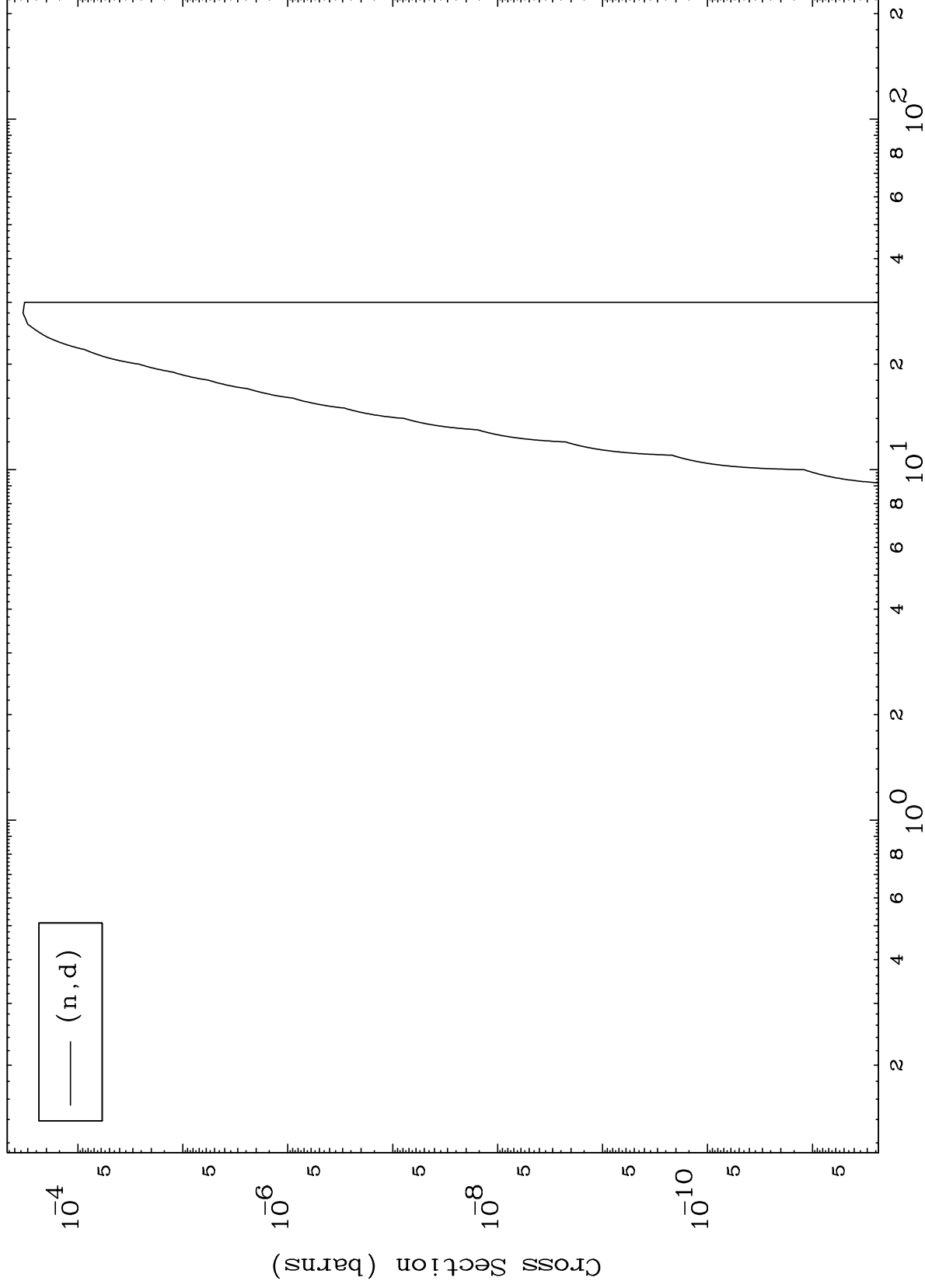
Incident Energy (MeV)

93-Np-237

MAT 9346

(He-3,d) Levels
0 Kelvin Cross Sections

93-Np-237



7

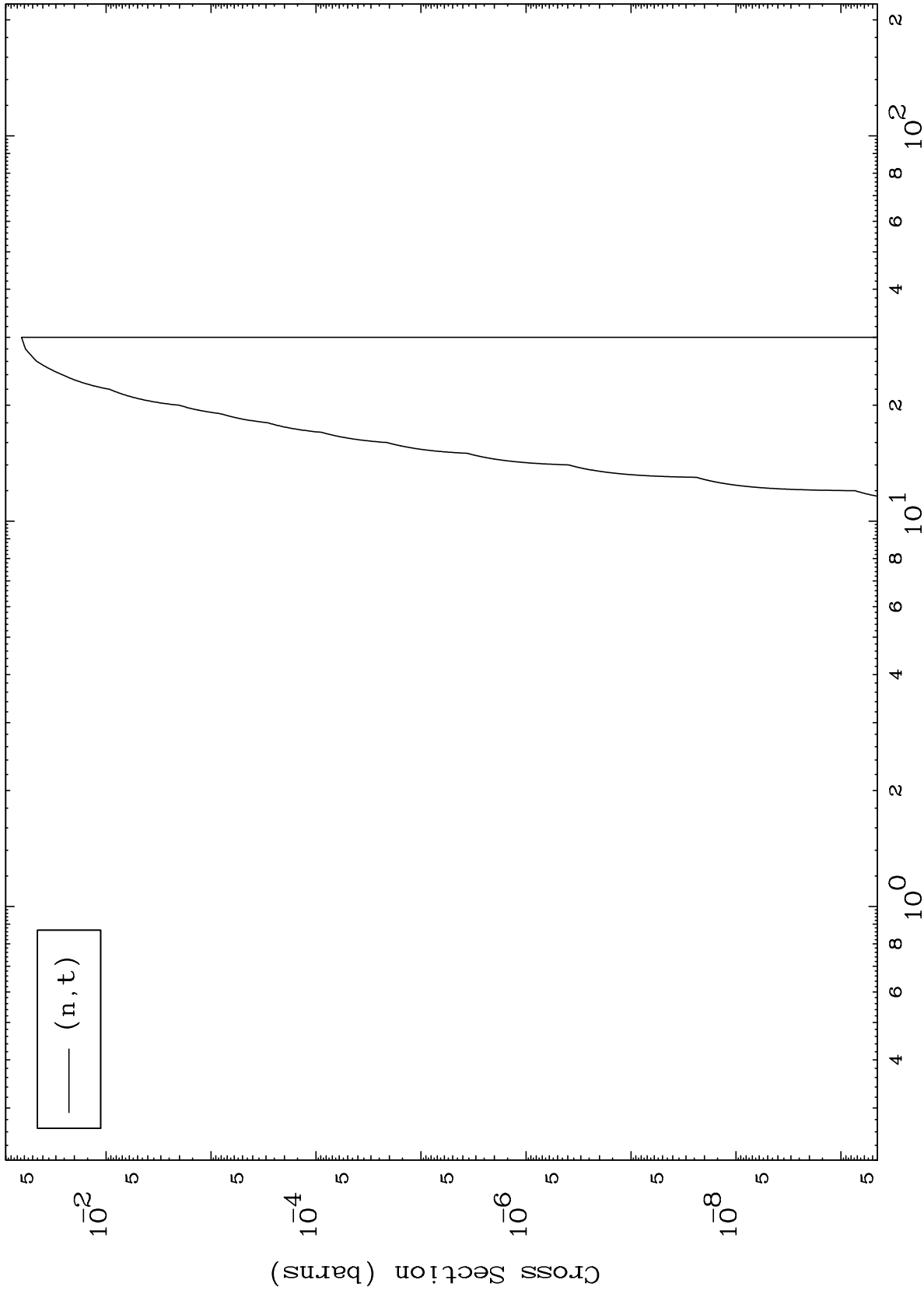
Incident Energy (MeV)

93-Np-237

MAT 9346

(He-3,t) Levels
0 Kelvin Cross Sections

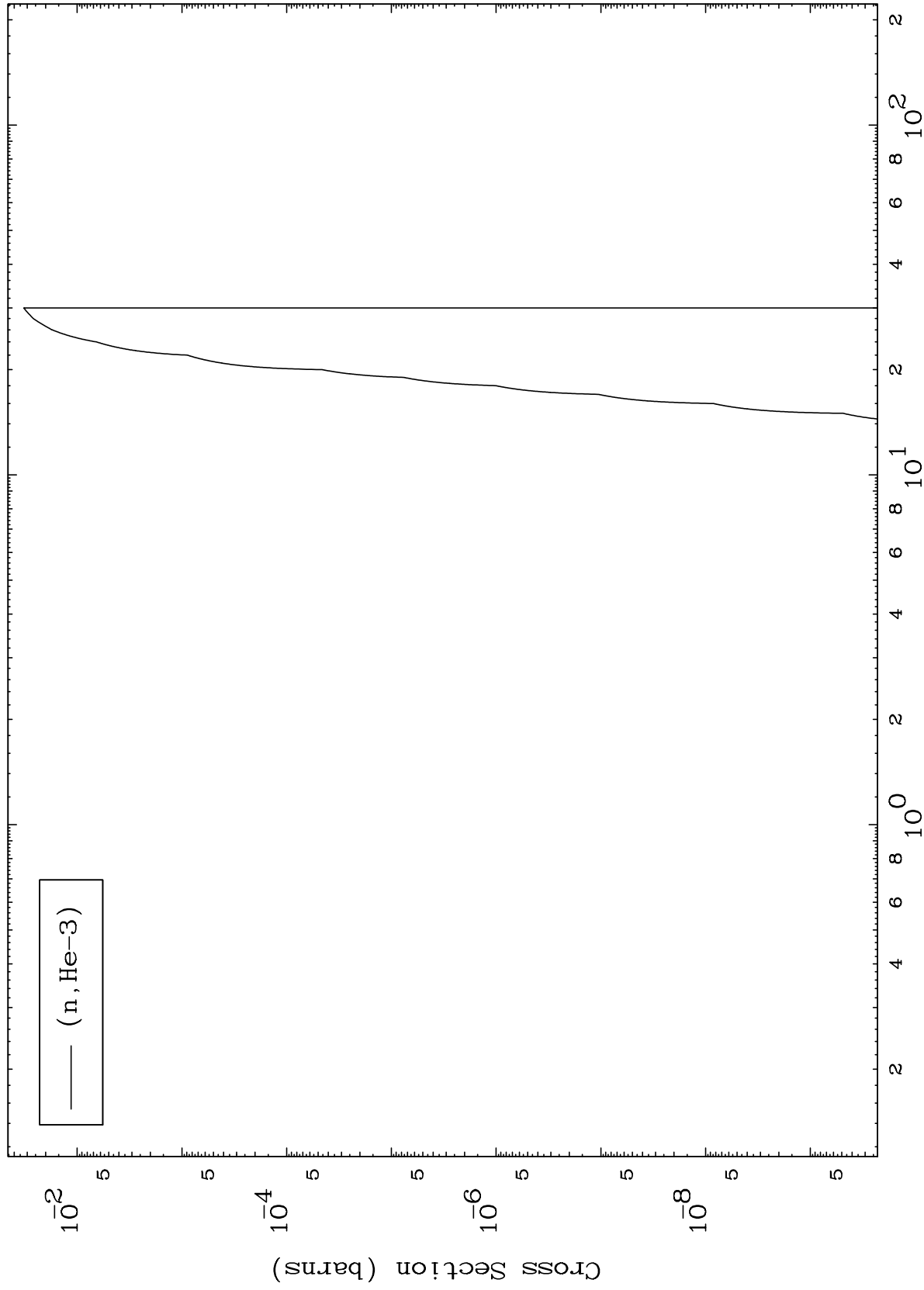
93-Np-237



8

Incident Energy (MeV)

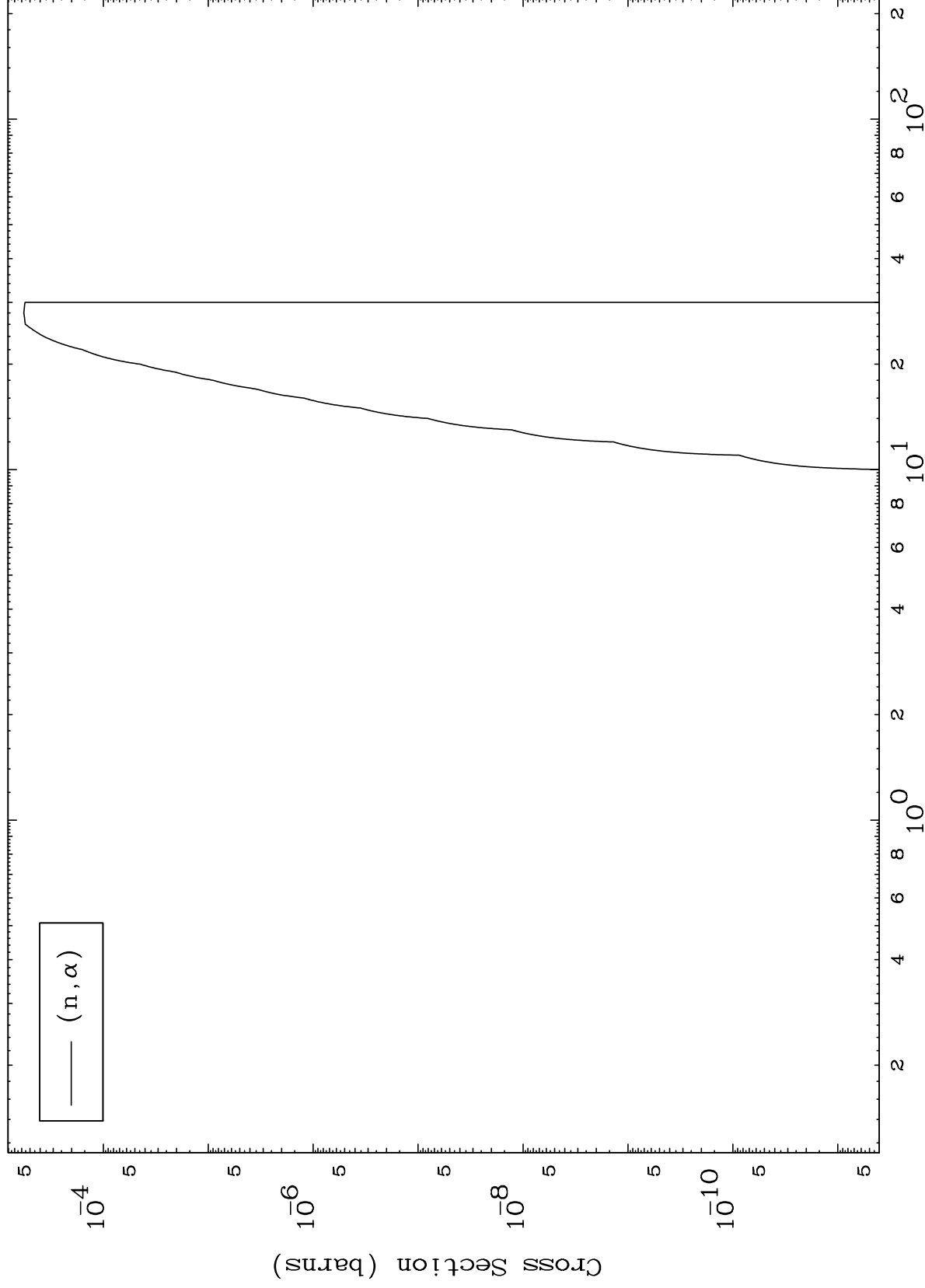
93-Np-237



MAT 9346

(He-3, α) Levels
0 Kelvin Cross Sections

93-Np-237



10

Incident Energy (MeV)

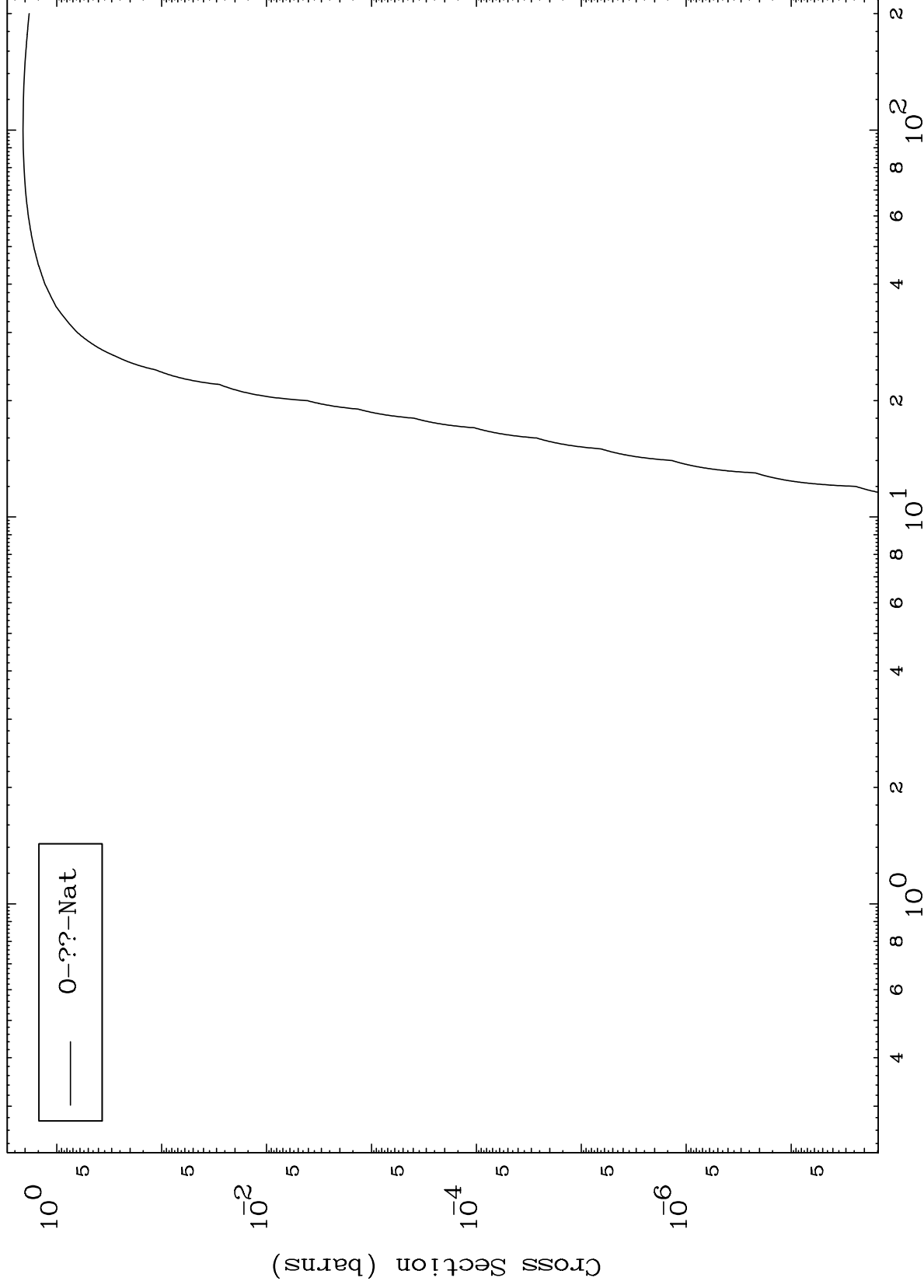
93-Np-237

MAT 9346

Fission

93-Np-237

Radionuclide Production Cross Section



11

Incident Energy (MeV)

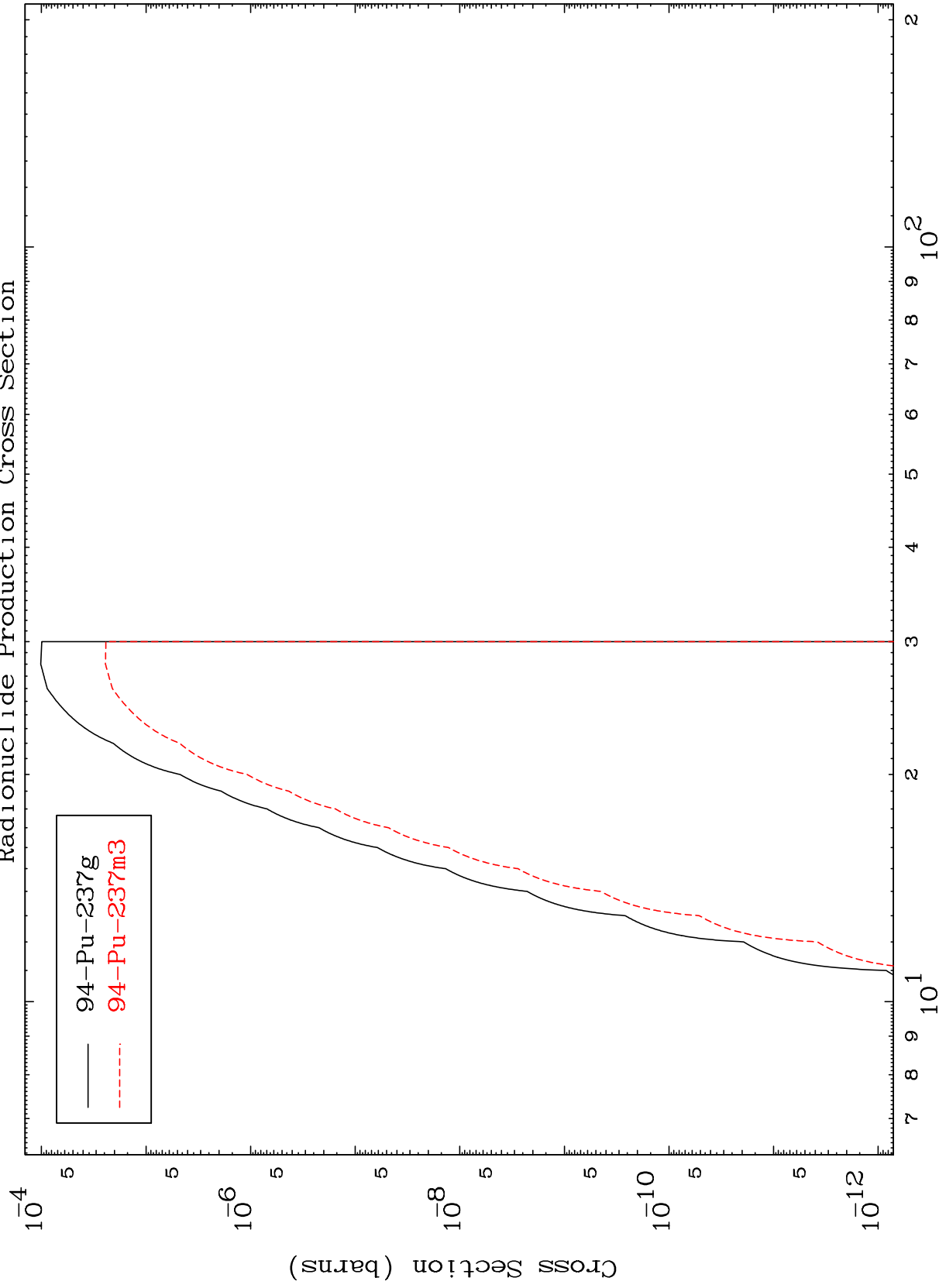
93-Np-237

MAT 9346

(n,n') d

93-Np-237

Radionuclide Production Cross Section



12

Incident Energy (MeV)

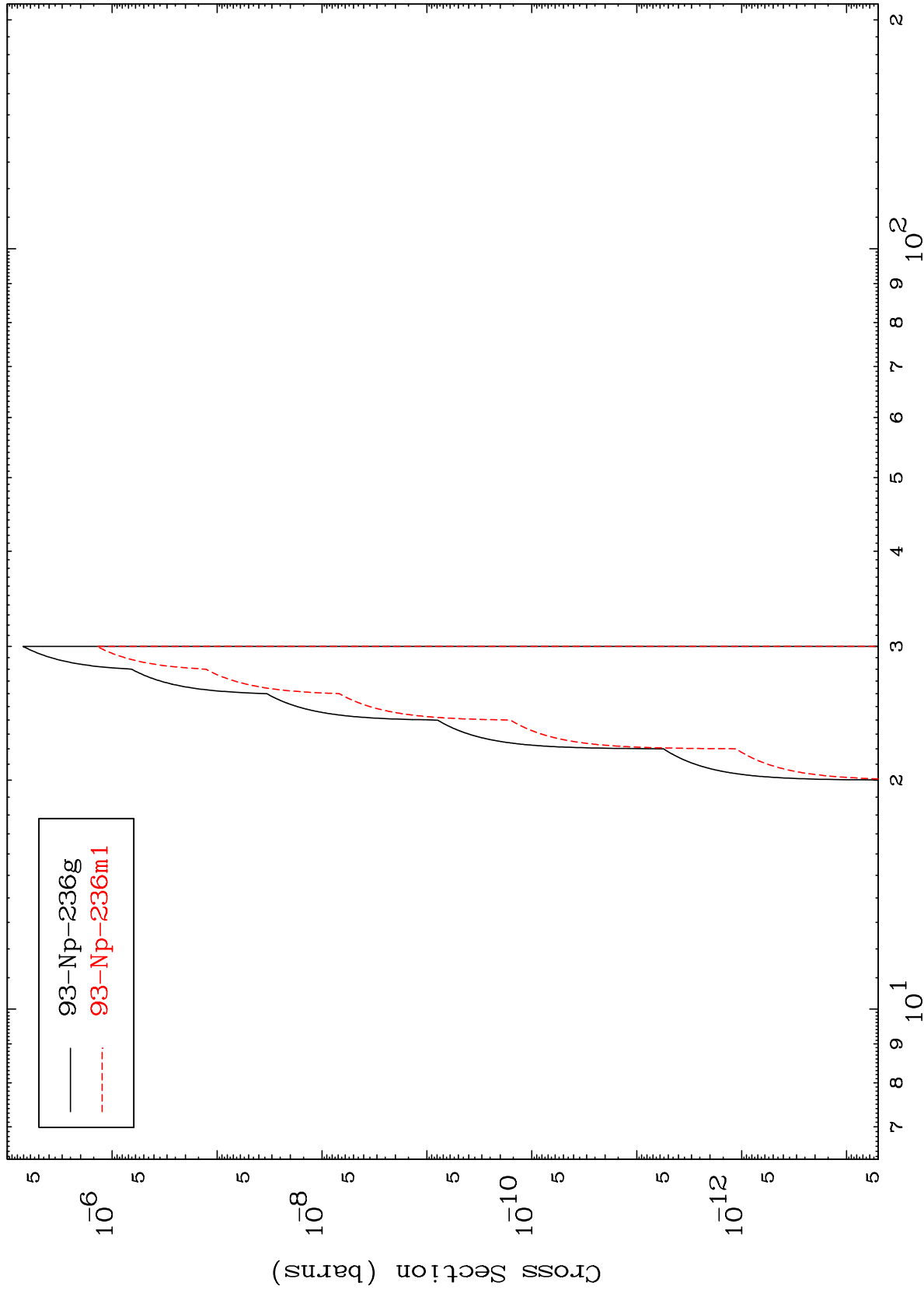
93-Np-237

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(n,n') He-3

93-Np-237

Radionuclide Production Cross Section



13

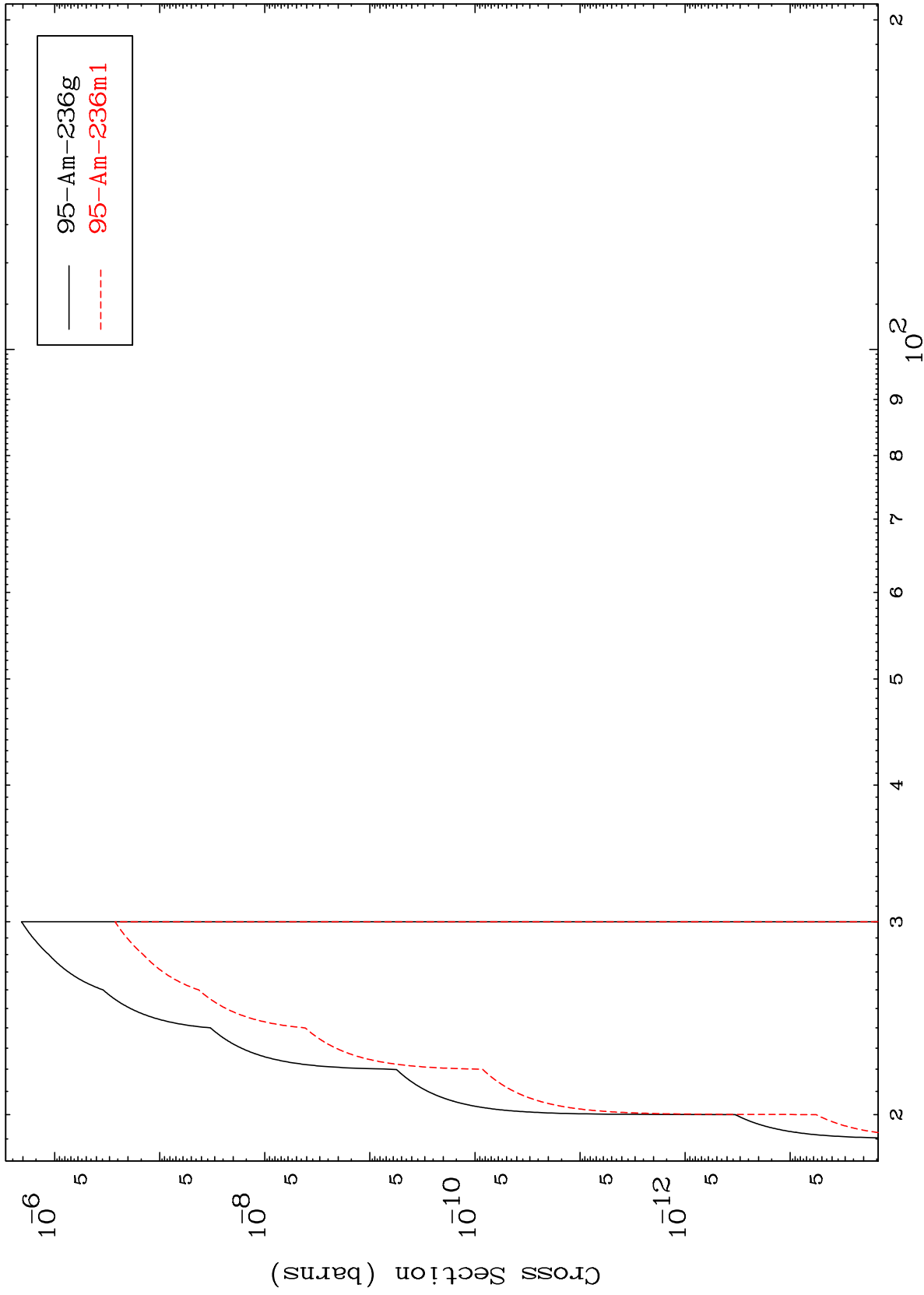
Incident Energy (MeV)

93-Np-237

MAT 9346

93-Np-237

(n,4n)
Radionuclide Production Cross Section



14

Incident Energy (MeV)

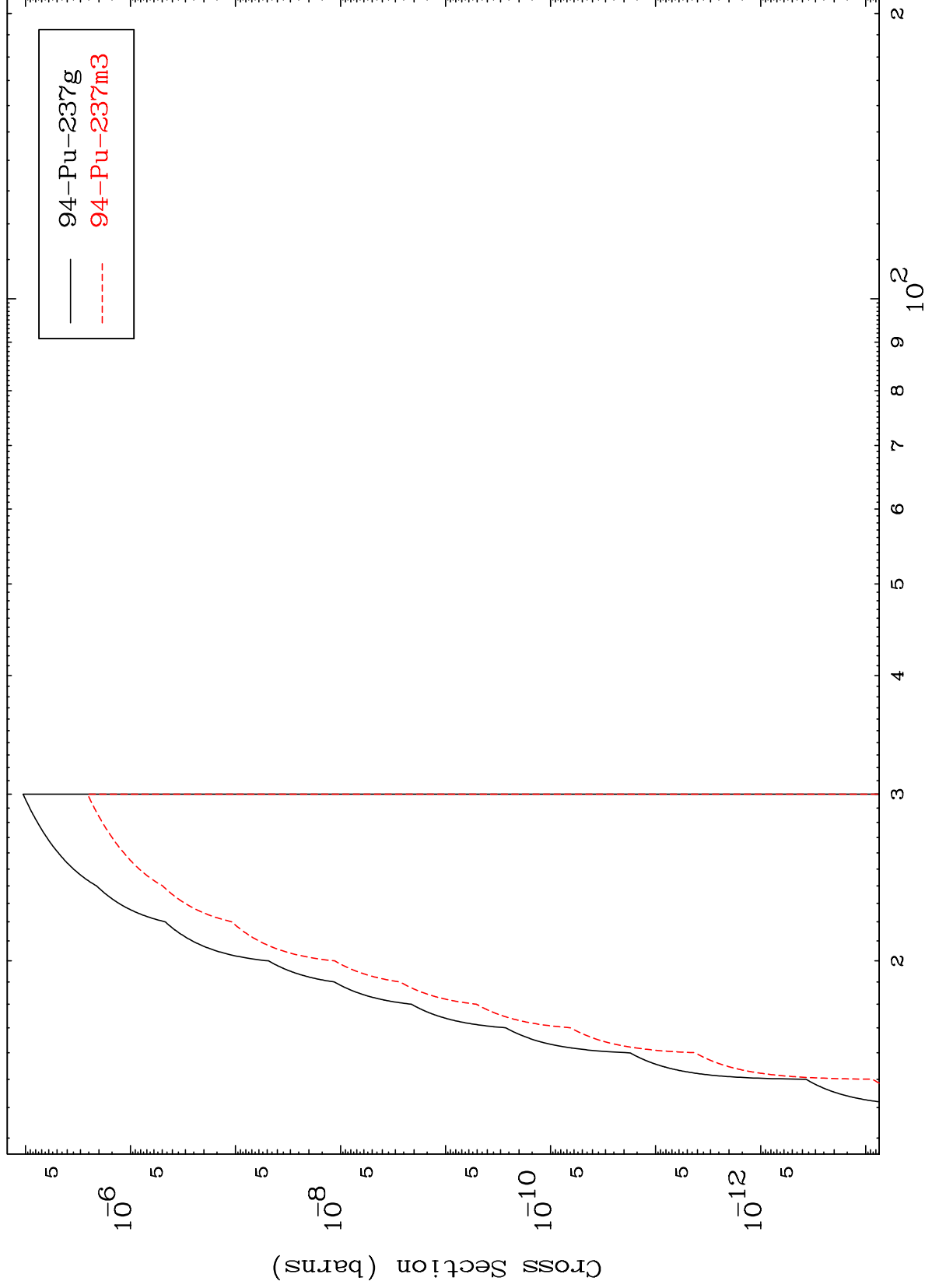
93-Np-237

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(n,2n) p

93-Np-237

Radionuclide Production Cross Section



15

Incident Energy (MeV)

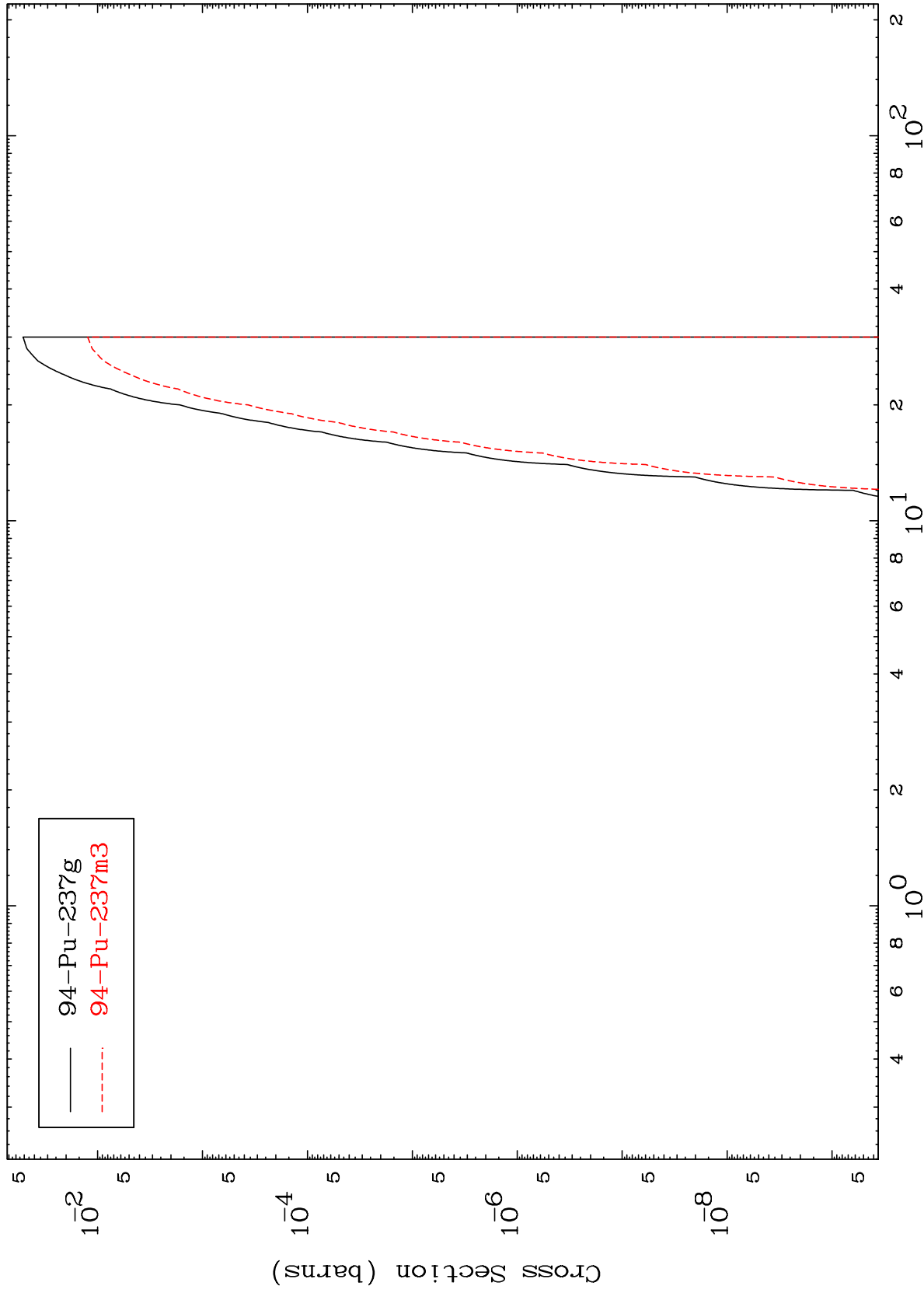
93-Np-237

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(n,t)

93-Np-237

Radionuclide Production Cross Section



16

Incident Energy (MeV)

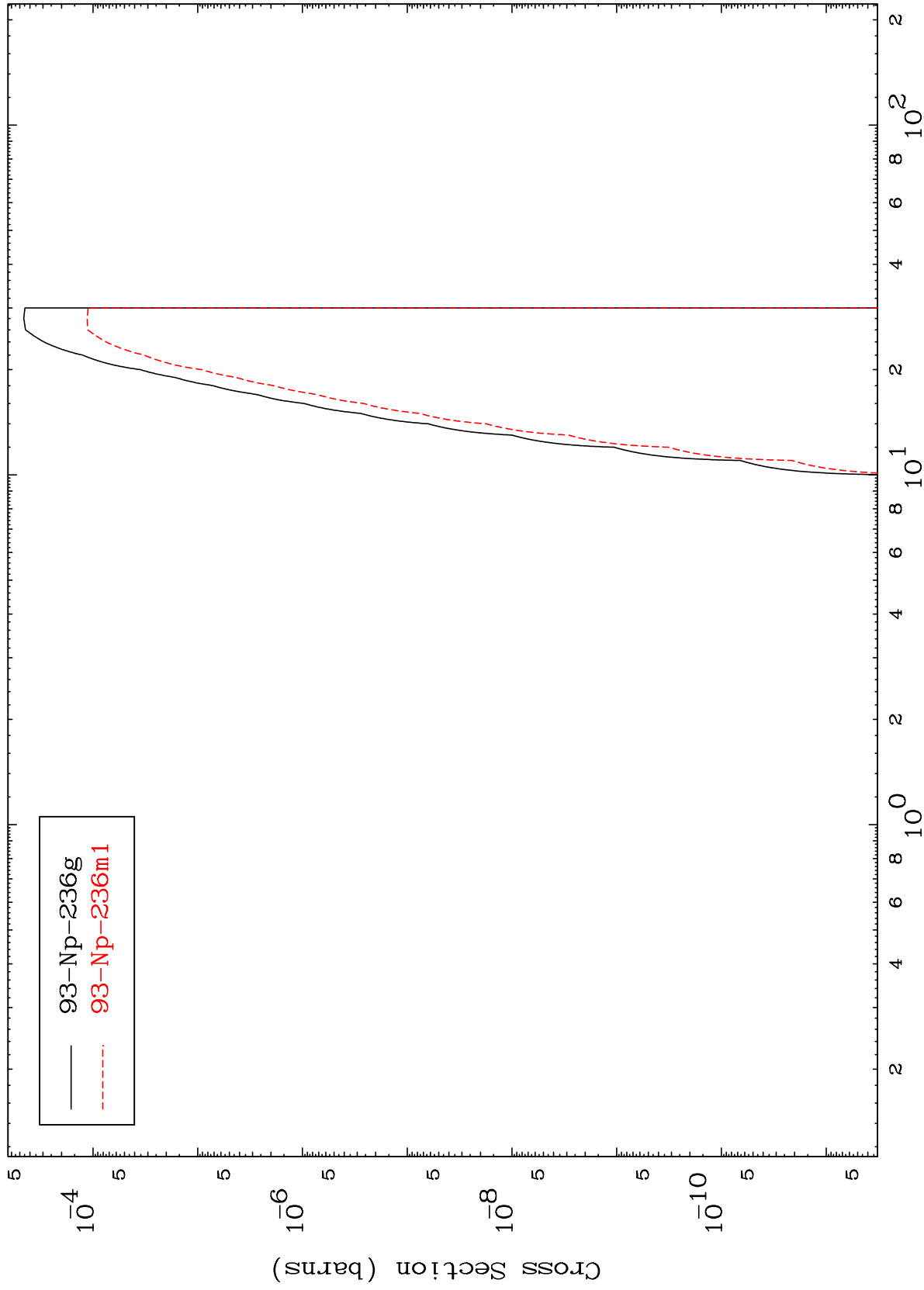
93-Np-237

MAT 9346

(n, α)

93-Np-237

Radionuclide Production Cross Section



17

Incident Energy (MeV)

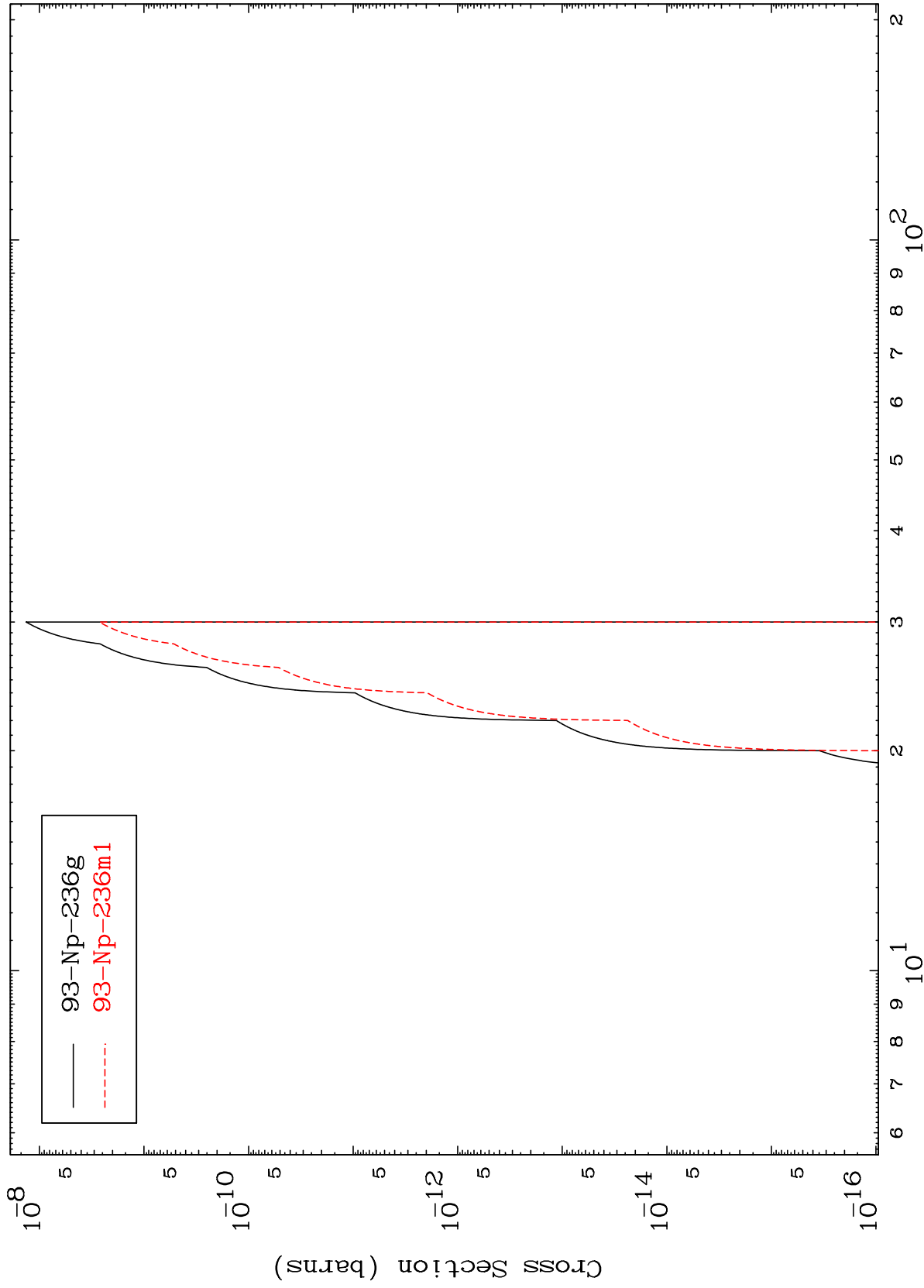
93-Np-237

MAT 9346

(n,p) t

93-Np-237

Radionuclide Production Cross Section



18

Incident Energy (MeV)

93-Np-237