

Program EVALPLOT  
(Version 2021-1)

by

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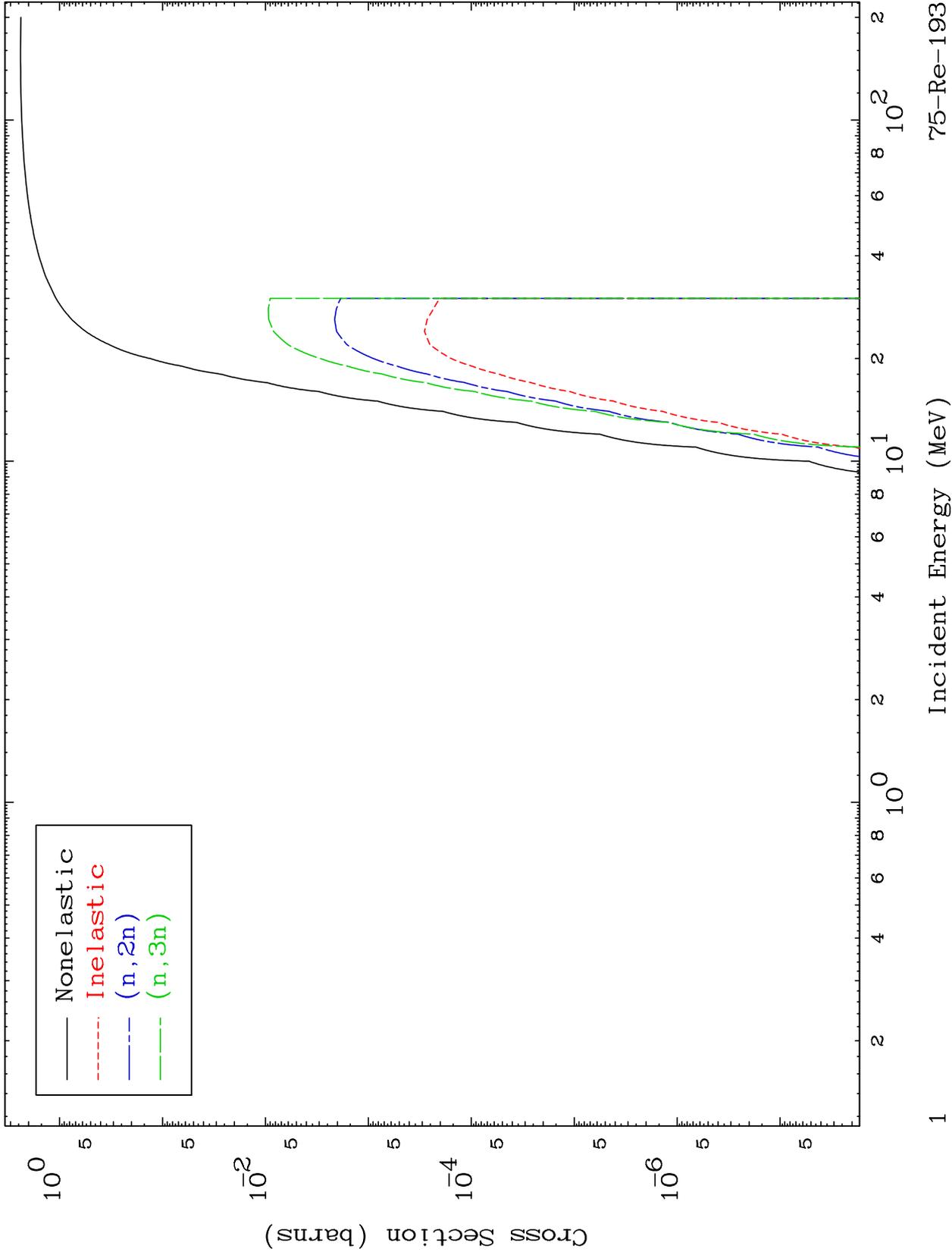
Press Mouse Button to Start

MAT 7549

He-3 Major

0 Kelvin Cross Sections

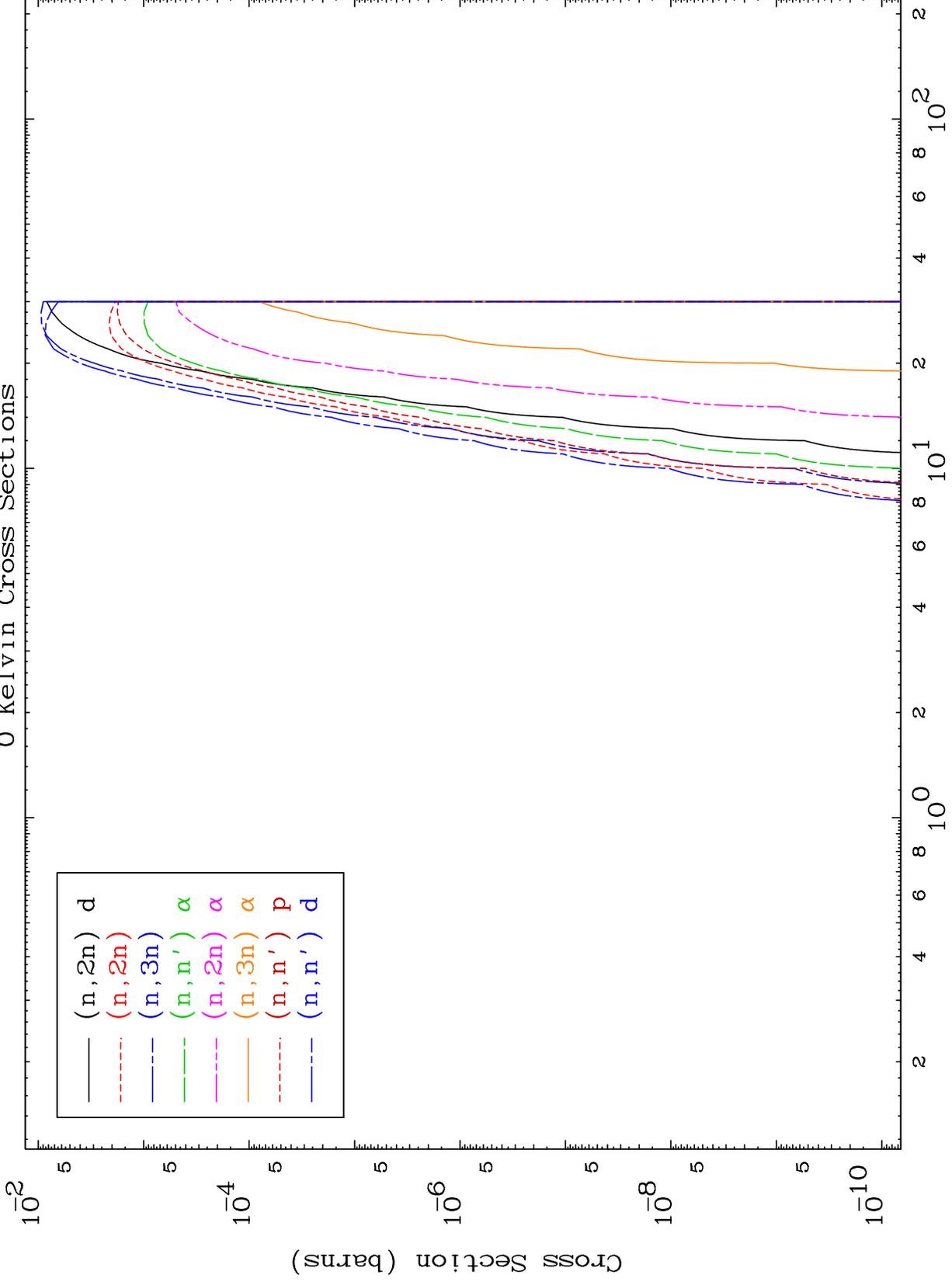
75-Re-193



MAT 7549

He-3 Neutron Absorption  
0 Kelvin Cross Sections

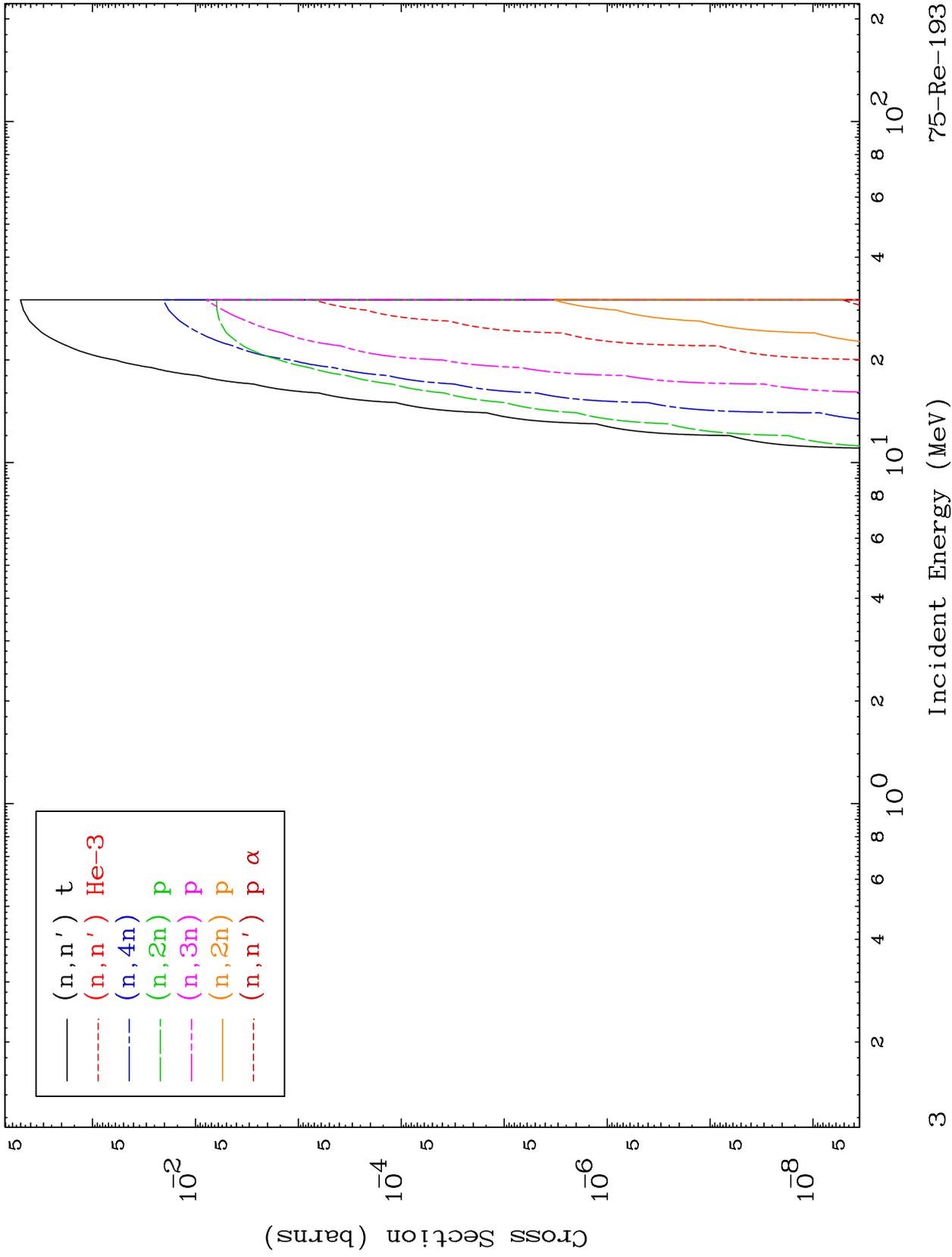
75-Re-193



MAT 7549

He-3 Neutron Absorption  
0 Kelvin Cross Sections

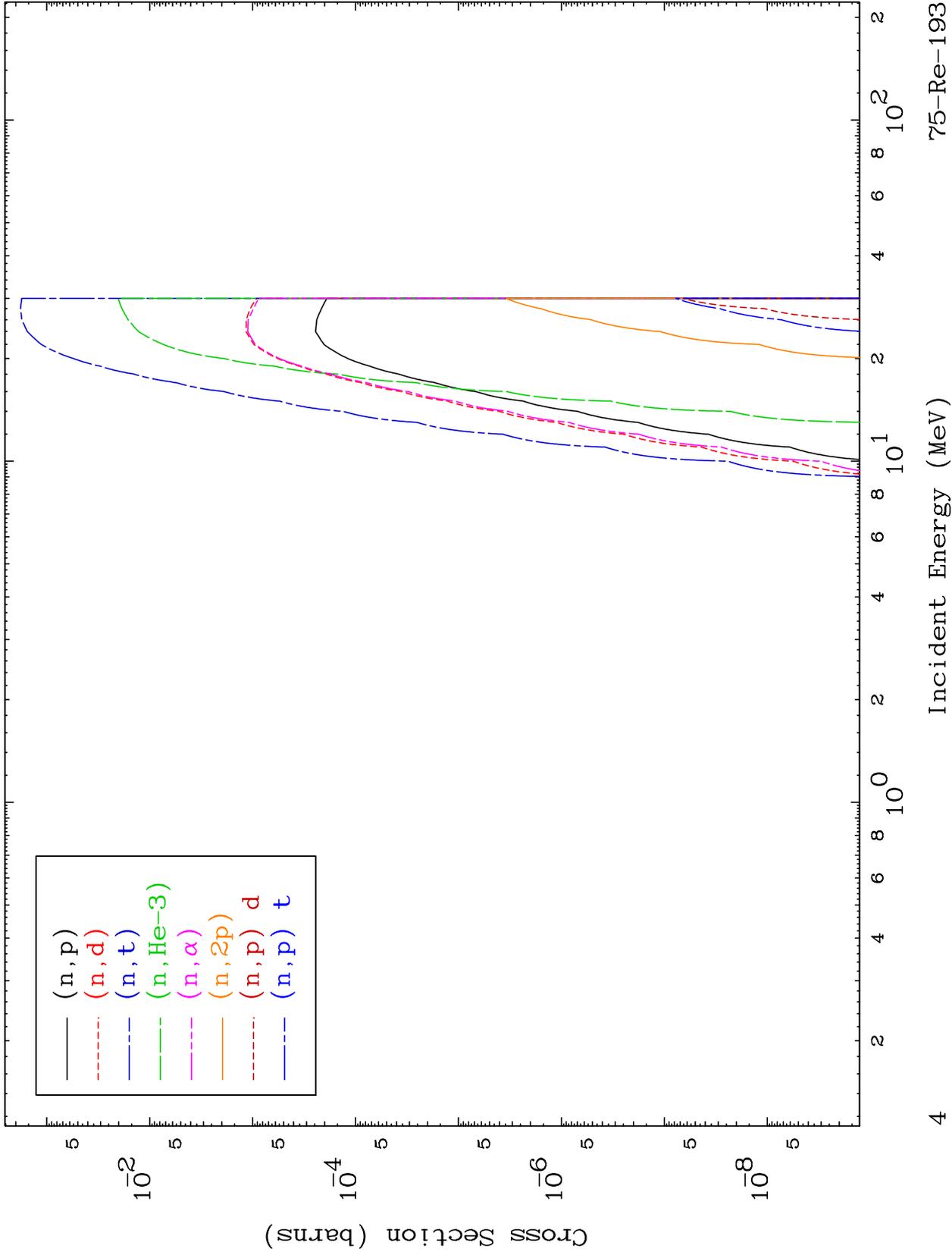
75-Re-193



MAT 7549

He-3 Neutron Absorption  
0 Kelvin Cross Sections

75-Re-193

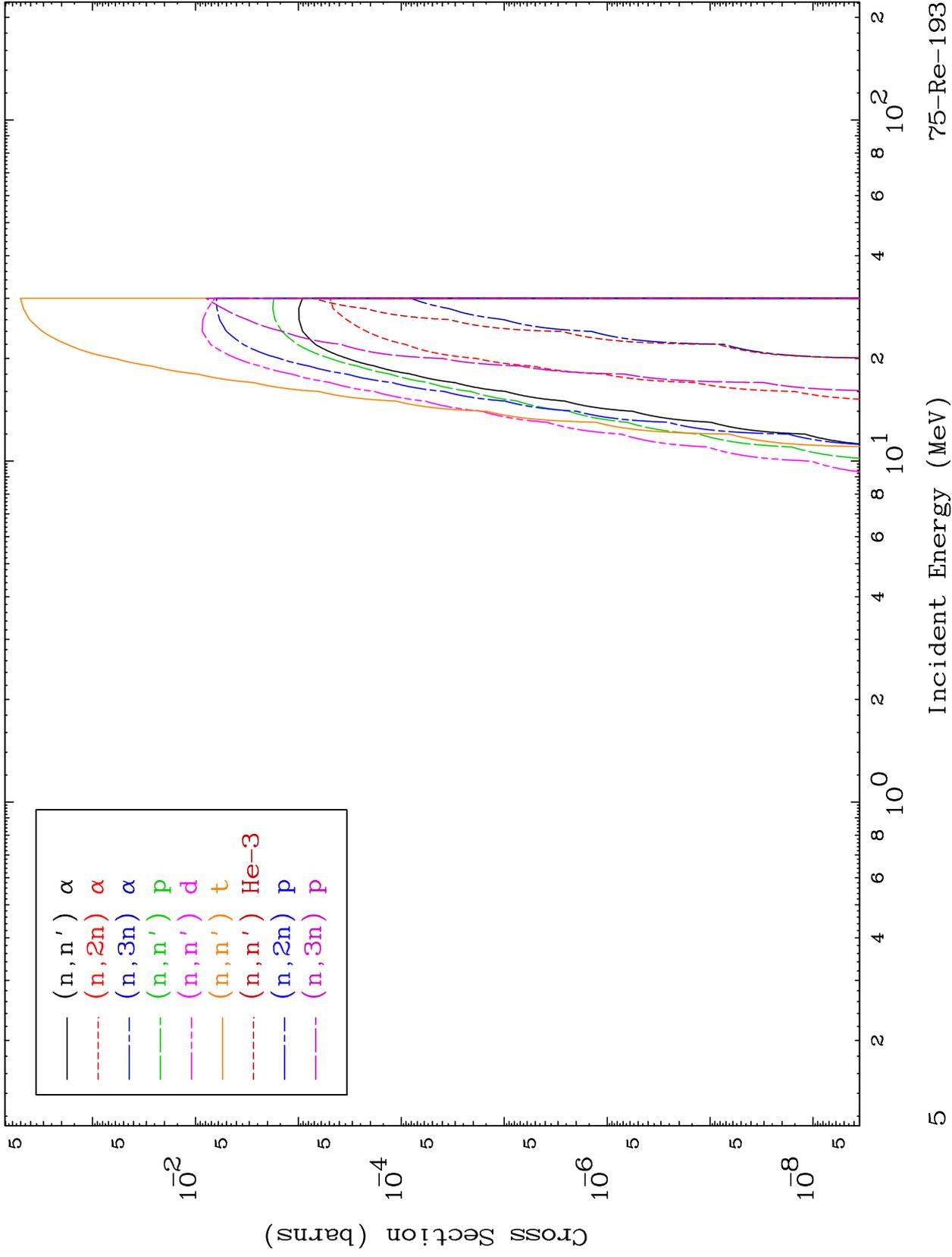


75-Re-193

MAT 7549

He-3 Charged Particle  
0 Kelvin Cross Sections

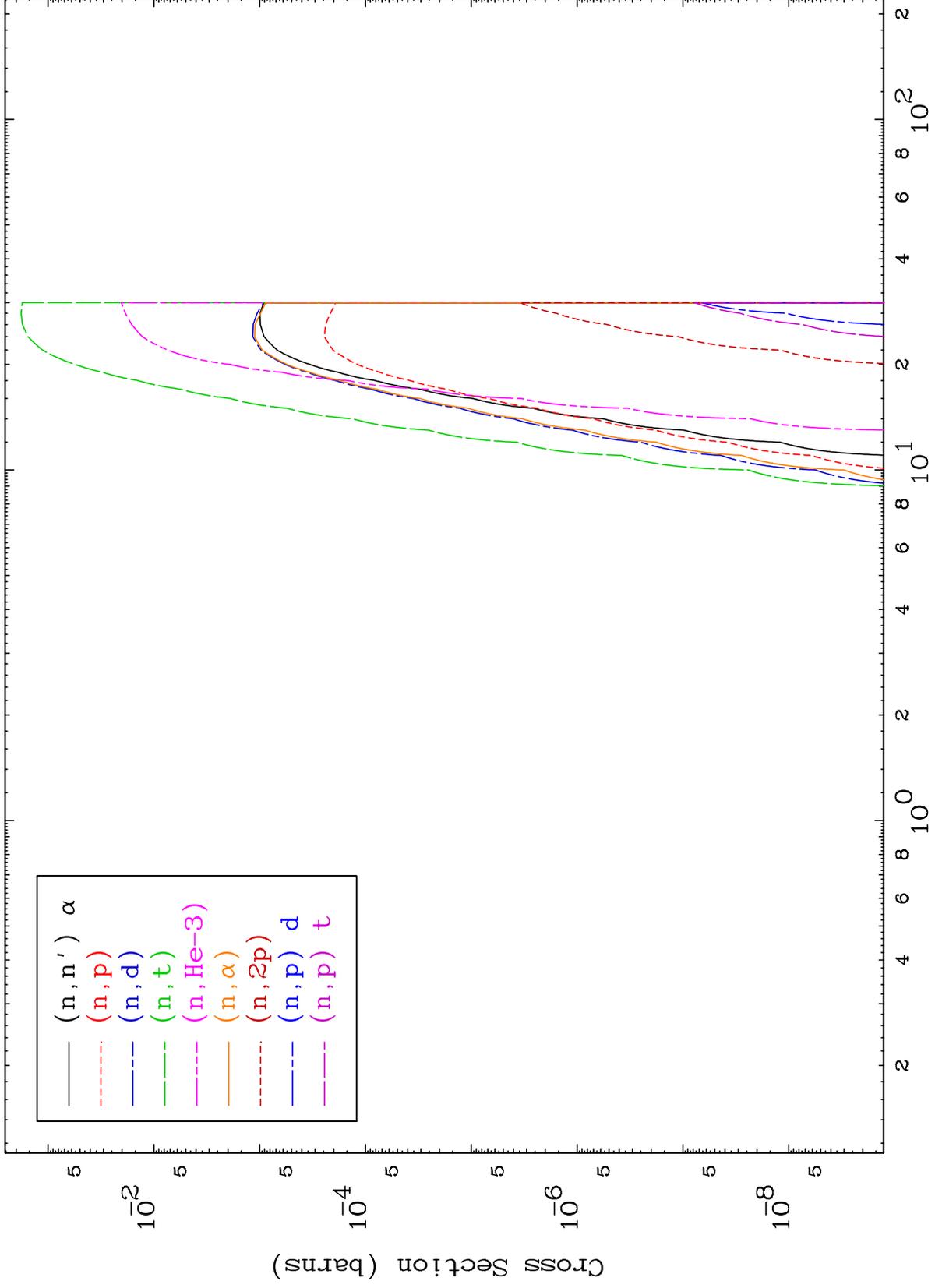
75-Re-193



MAT 7549

He-3 Charged Particle  
0 Kelvin Cross Sections

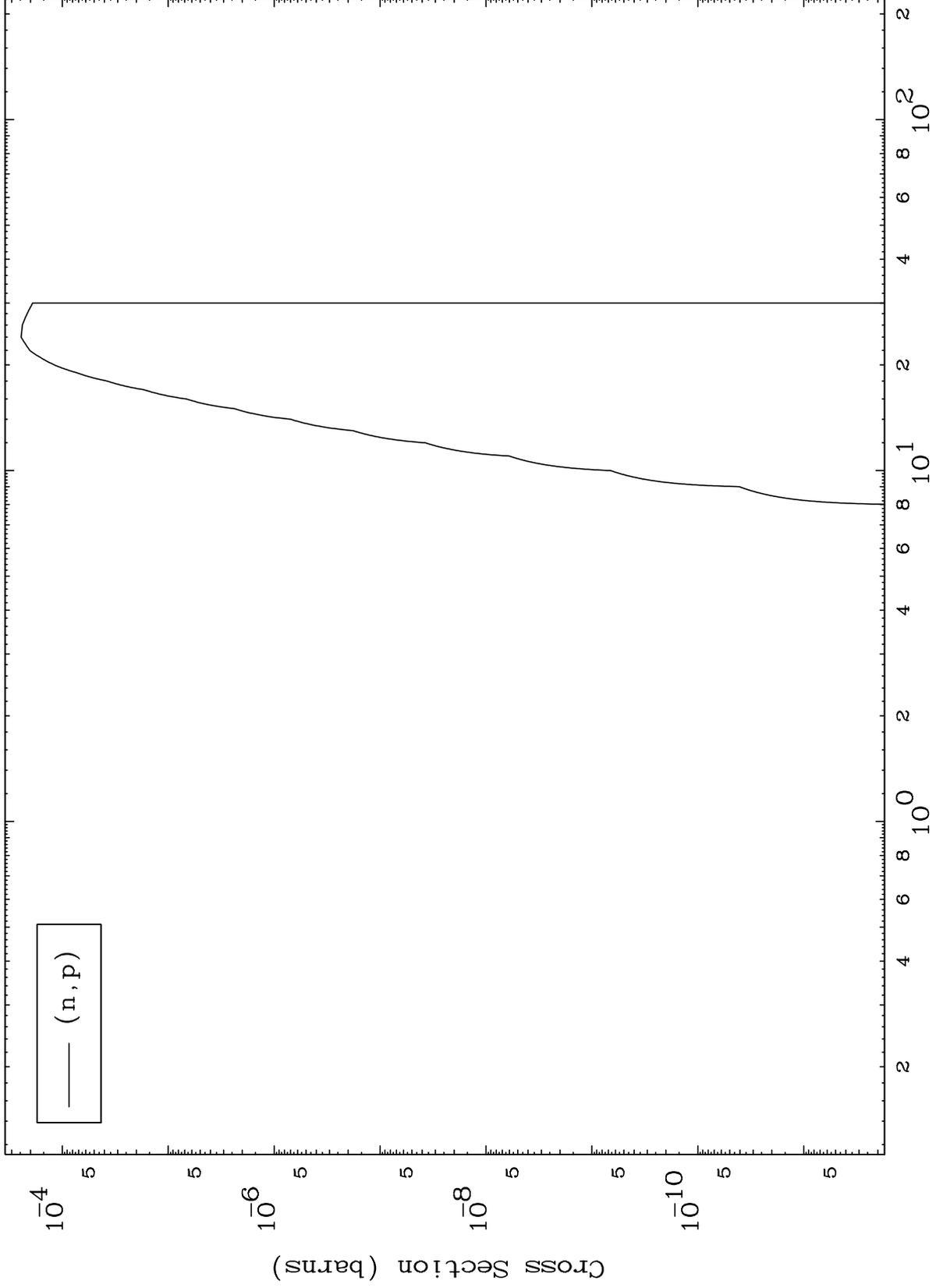
75-Re-193



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(He-3,p) Levels  
0 Kelvin Cross Sections

75-Re-193

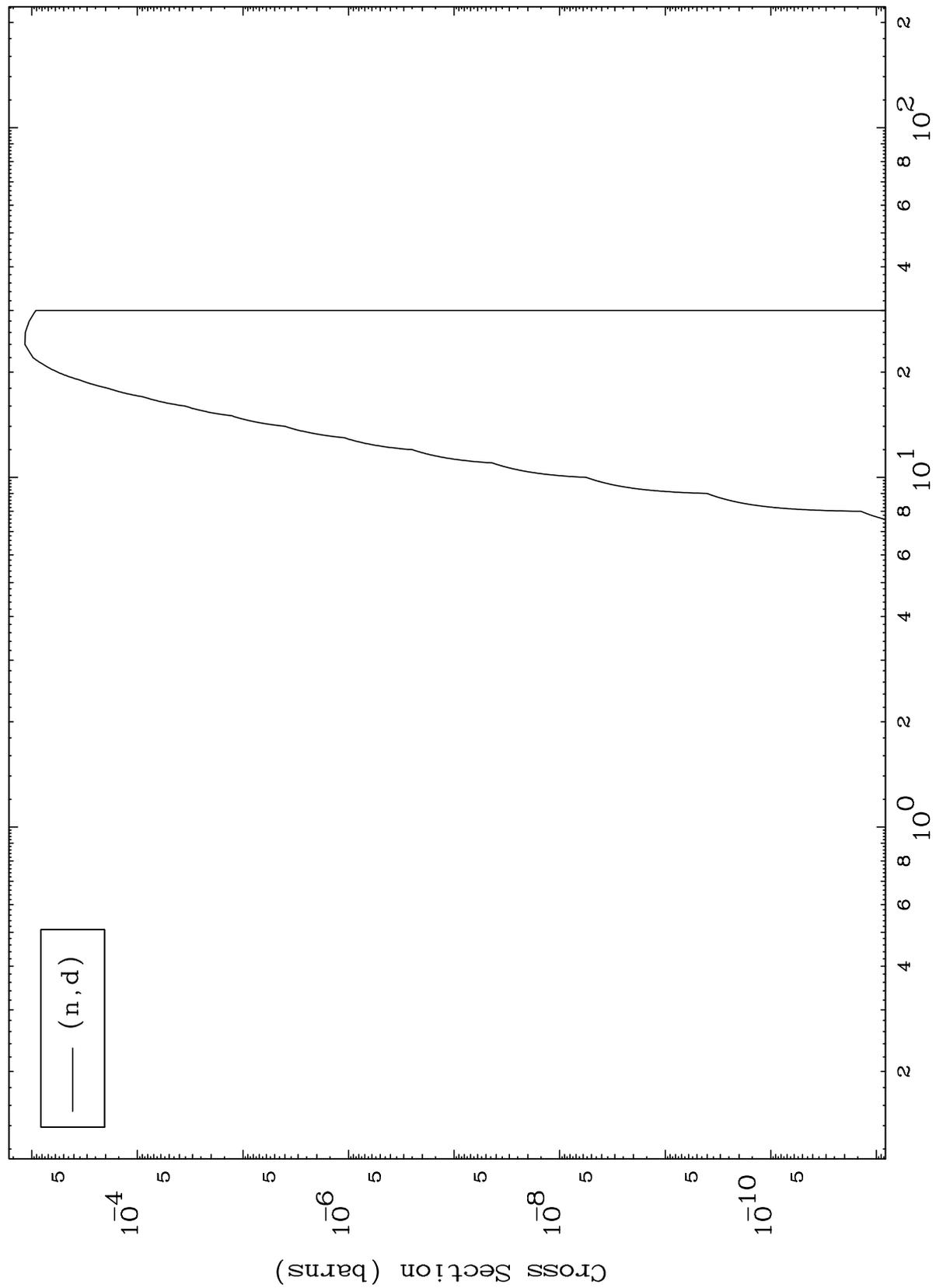


MAT 7549

(He-3,d) Levels

75-Re-193

0 Kelvin Cross Sections

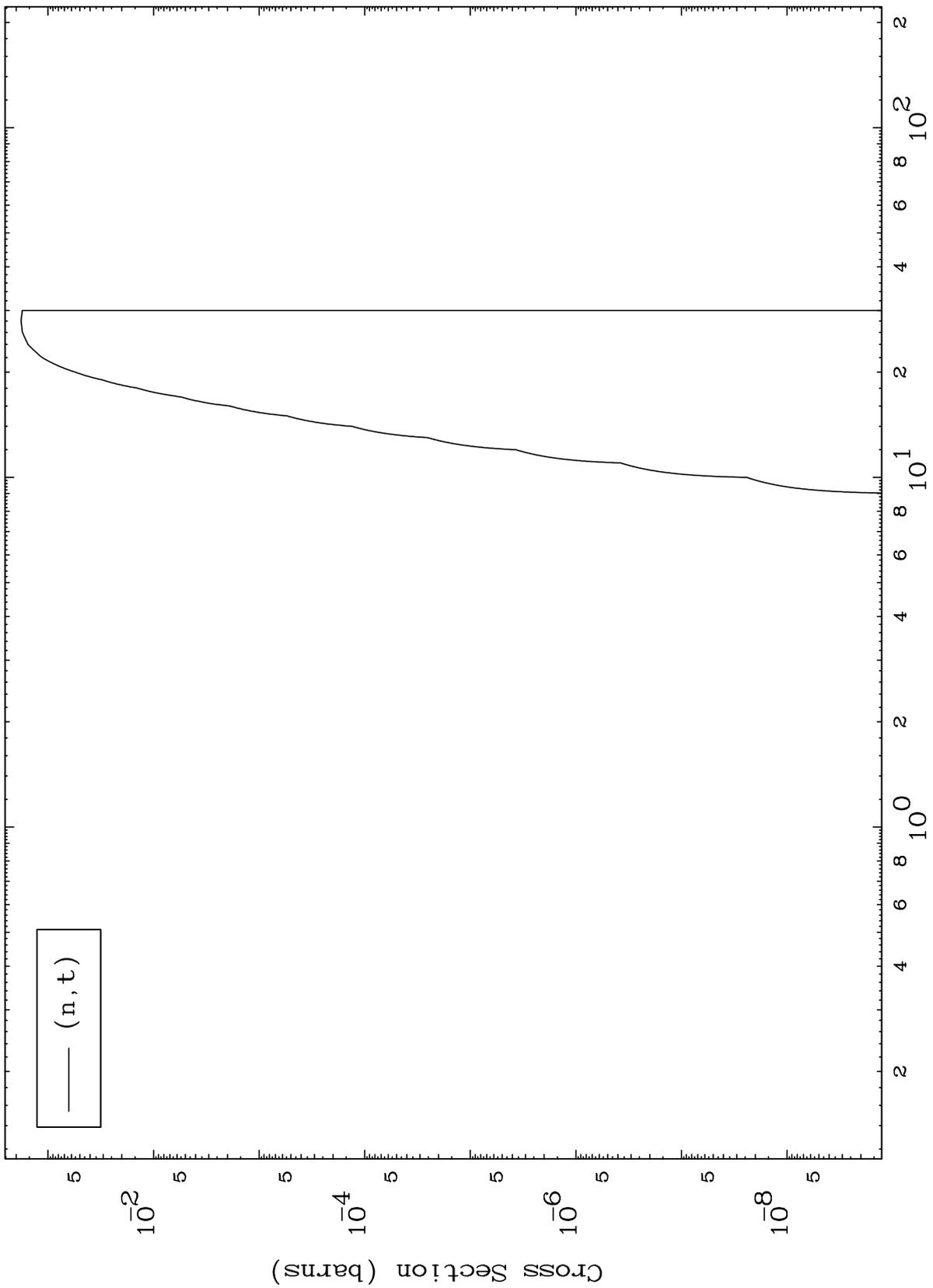


MAT 7549

(He-3,t) Levels

75-Re-193

0 Kelvin Cross Sections

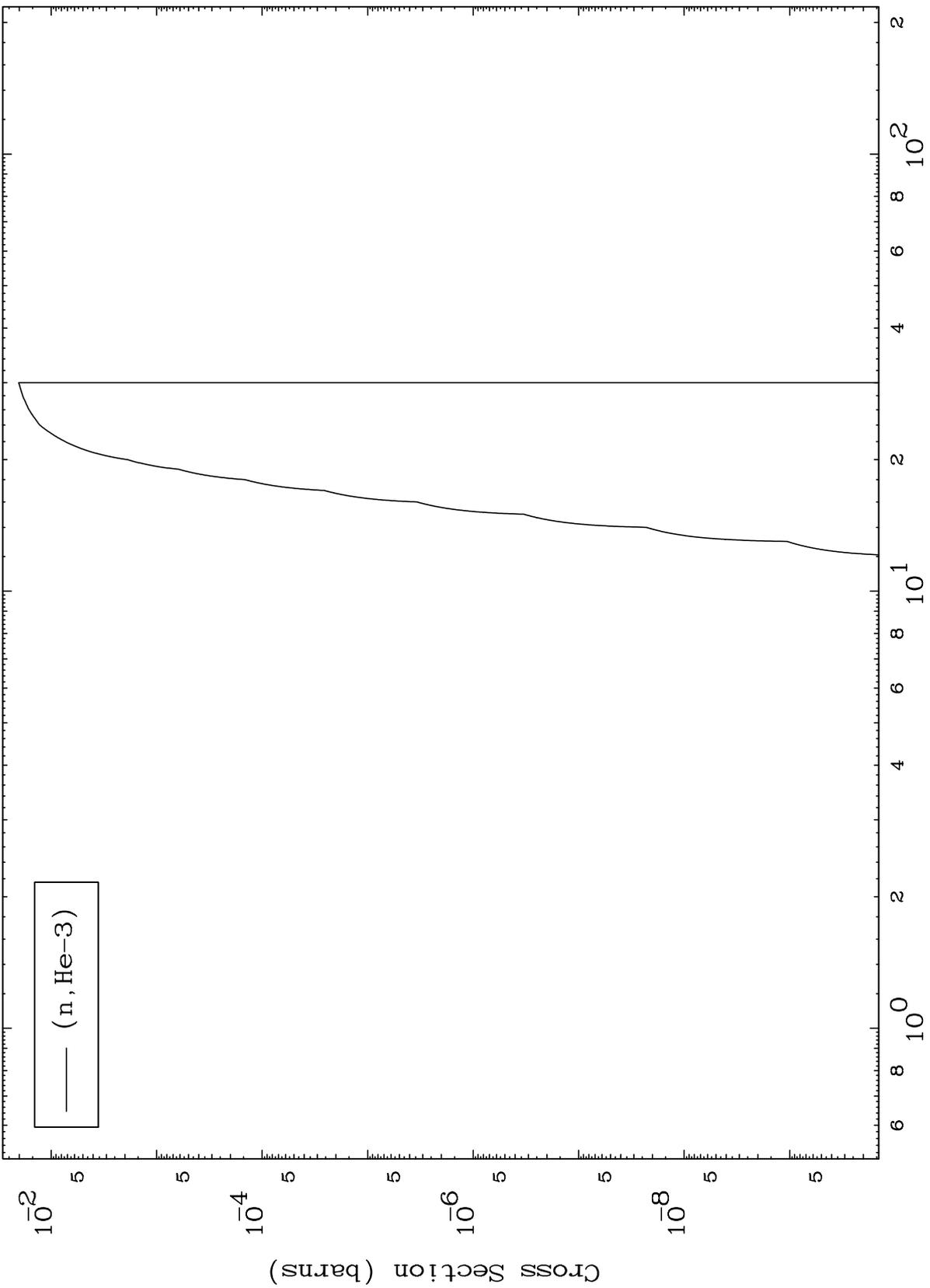


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(He-3, He3) Levels

75-Re-193

0 Kelvin Cross Sections



10

Incident Energy (MeV)

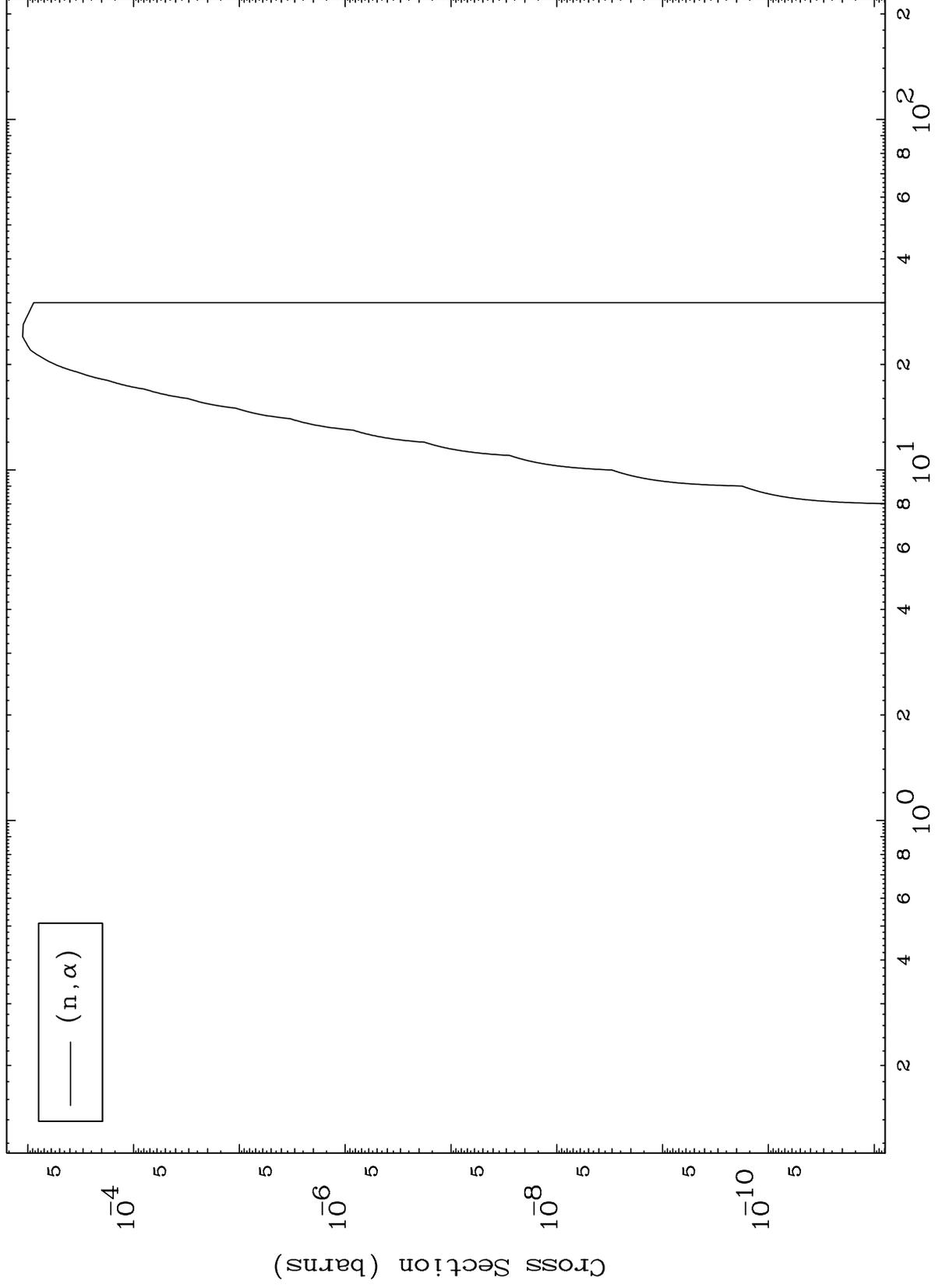
75-Re-193

MAT 7549

(He-3,  $\alpha$ ) Levels

75-Re-193

0 Kelvin Cross Sections



11

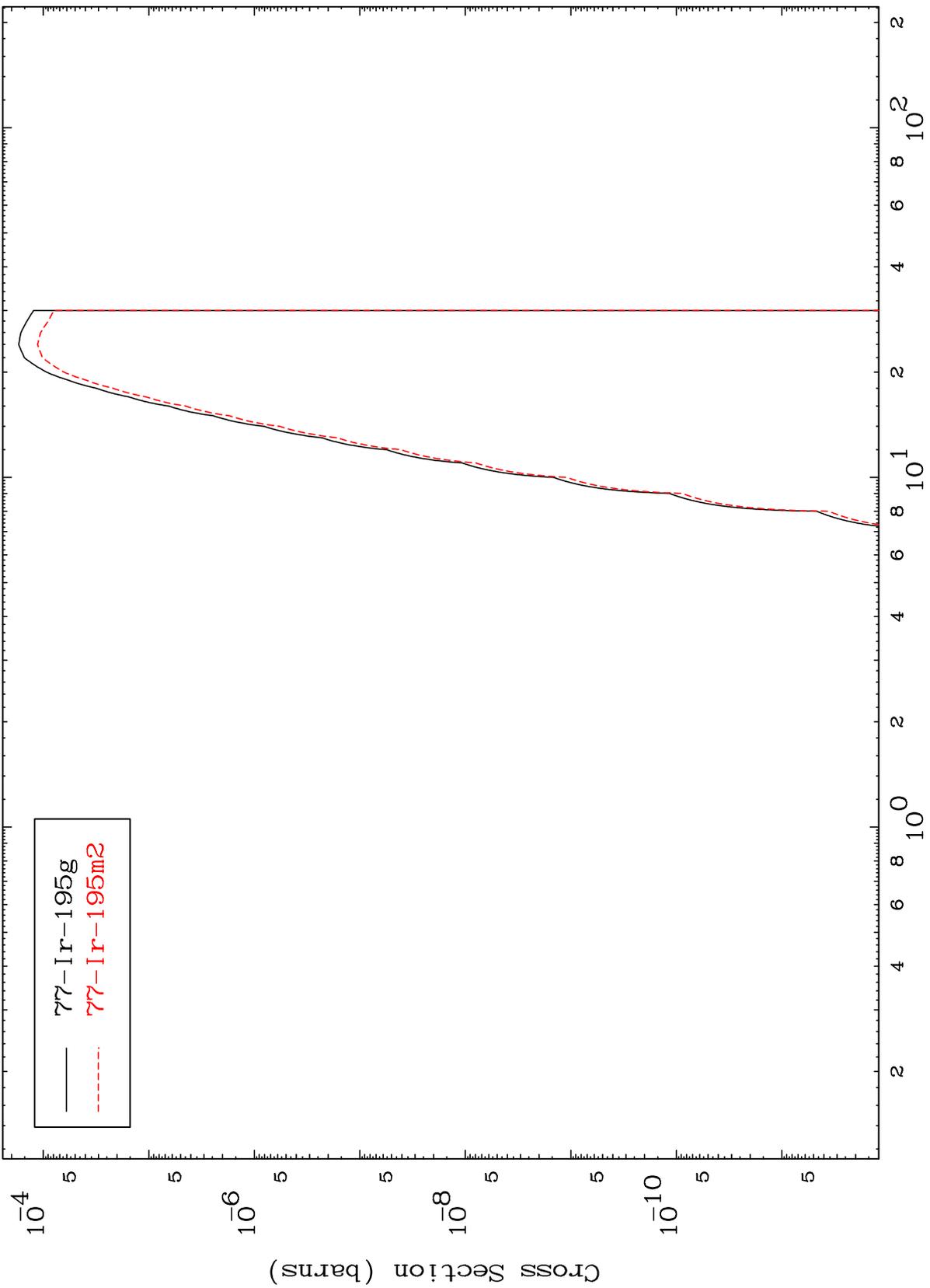
Incident Energy (MeV)

75-Re-193

MAT 7549

75-Re-193

Inelastic  
Radionuclide Production Cross Section



75-Re-193

Incident Energy (MeV)

12

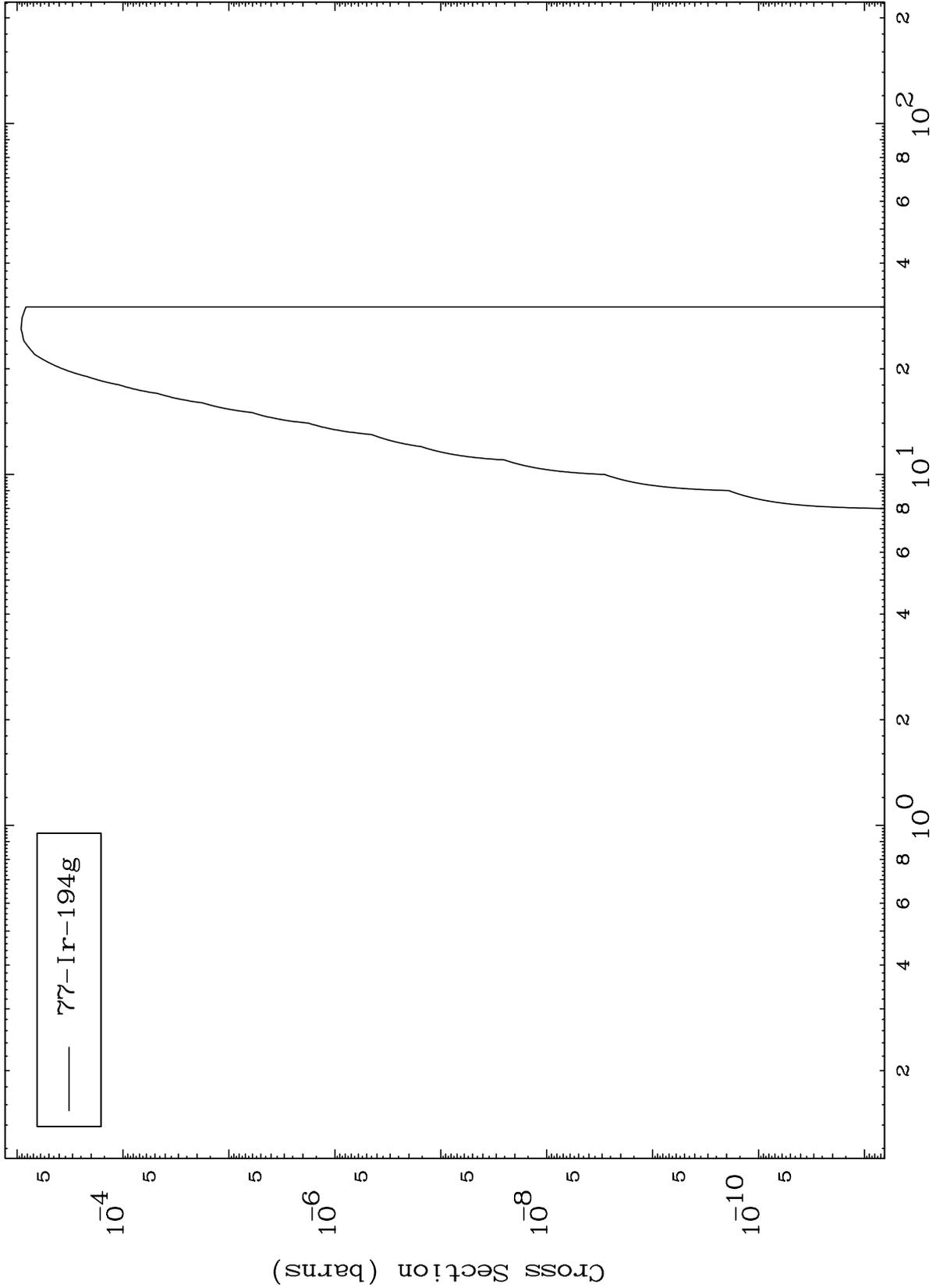


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(n,2n)

75-Re-193

Radionuclide Production Cross Section



14

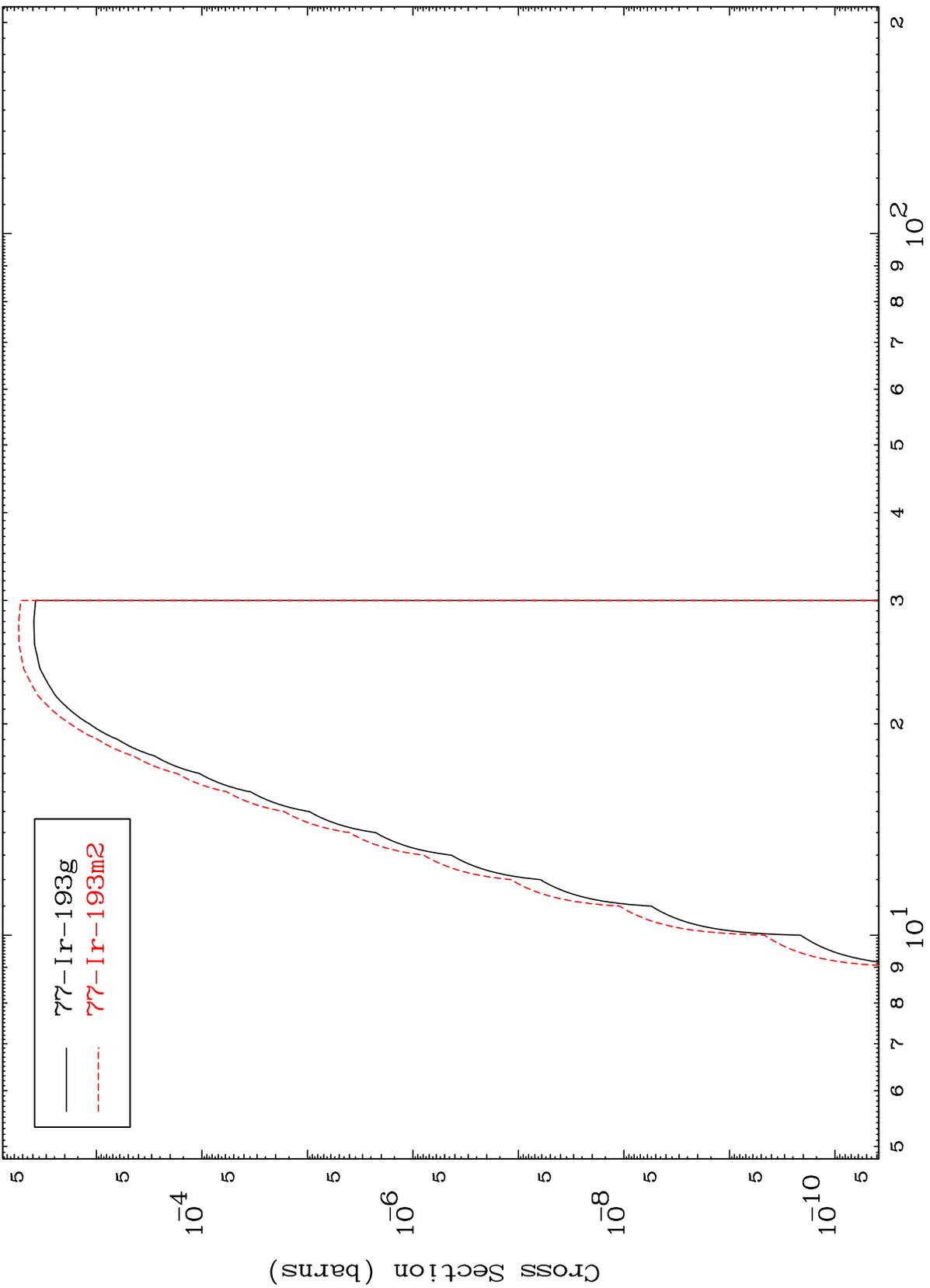
Incident Energy (MeV)

75-Re-193

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75-Re-193

Radionuclide Production Cross Section (n,3n)



Legend:  
—  $^{77}\text{Ir-193g}$   
- - -  $^{77}\text{Ir-193m2}$

15

Incident Energy (MeV)

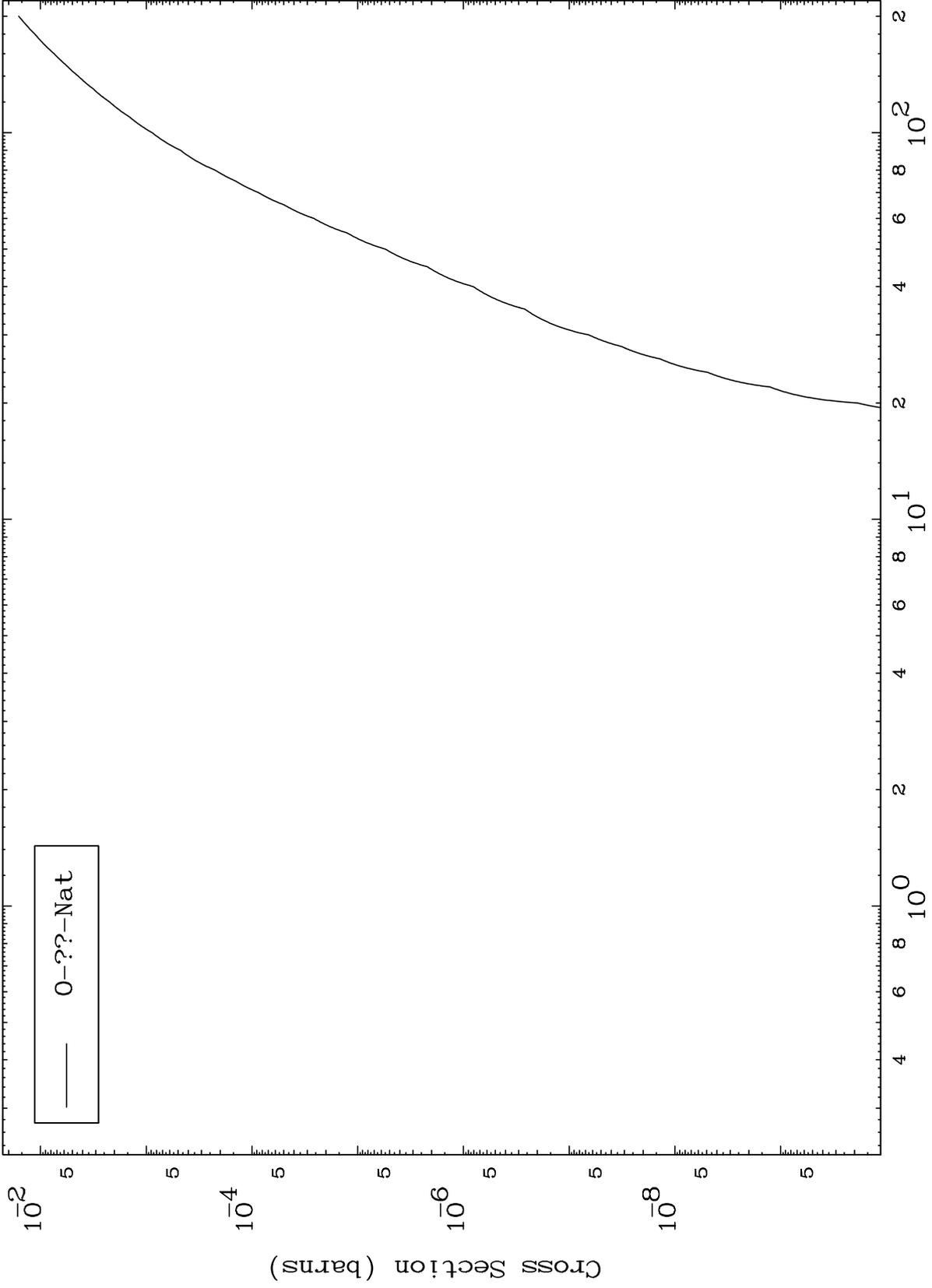
75-Re-193

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Fission

75-Re-193

Radionuclide Production Cross Section



16

Incident Energy (MeV)

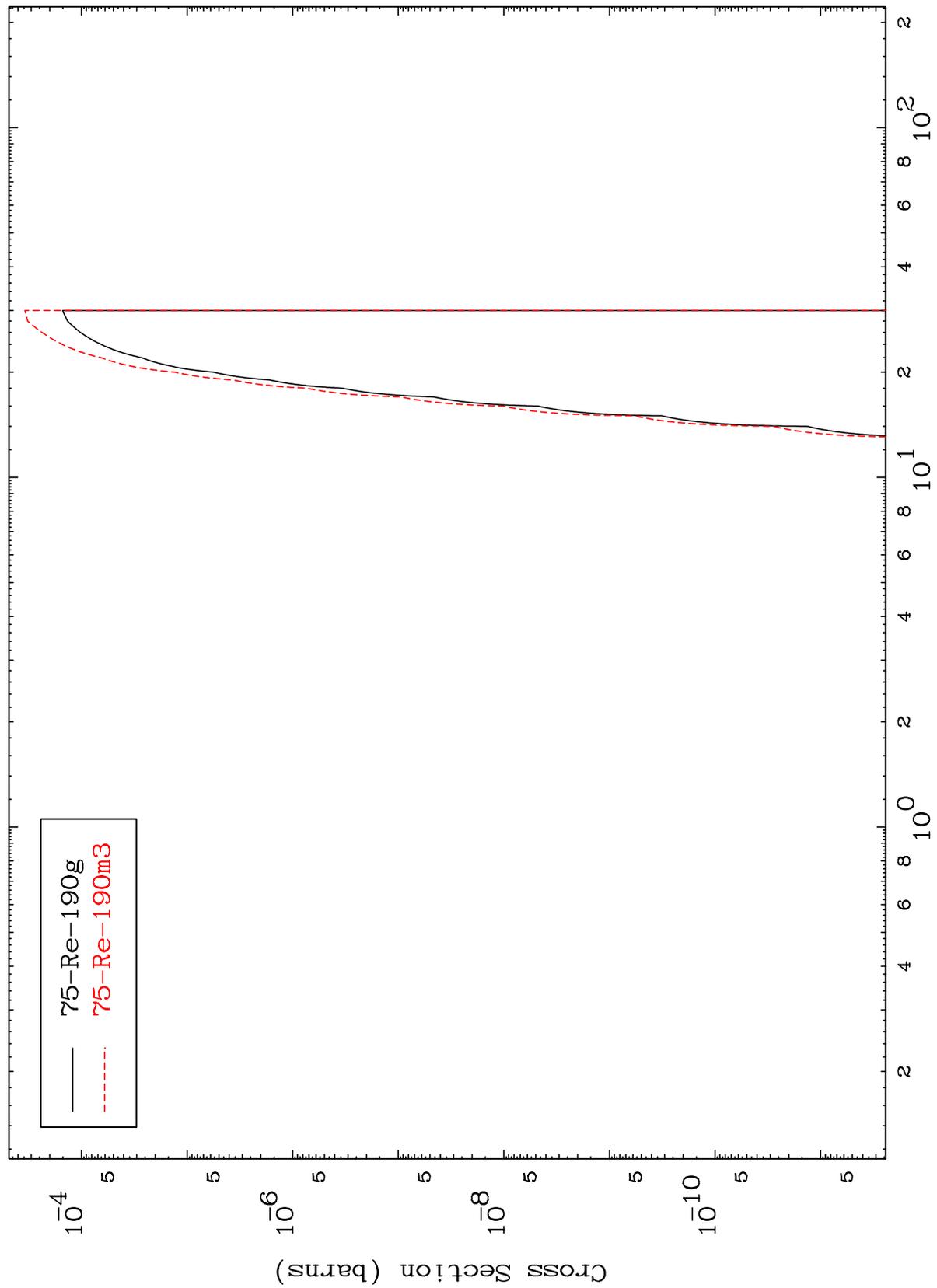
75-Re-193

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(n,2n)  $\alpha$

75-Re-193

Radionuclide Production Cross Section

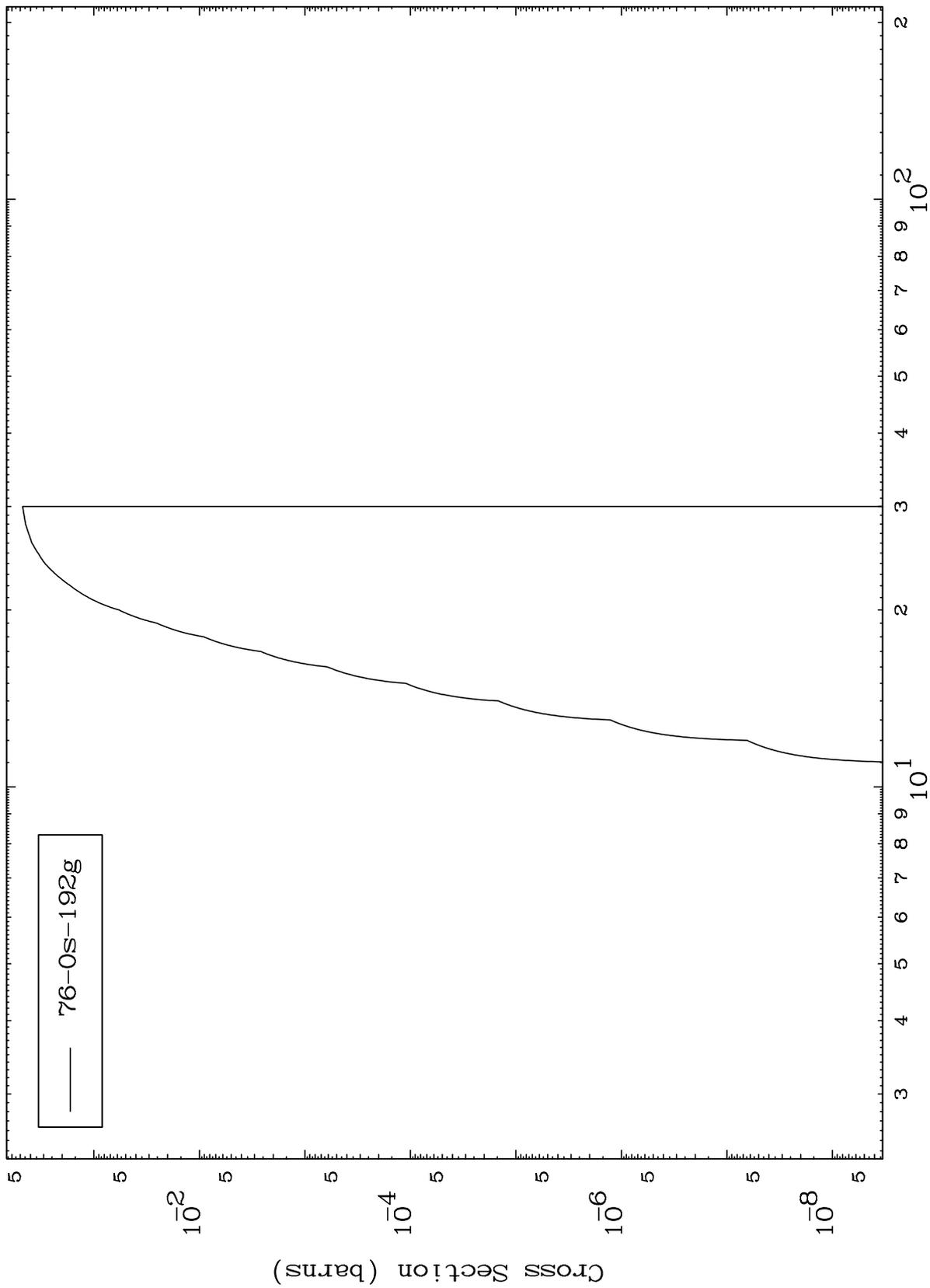


MAT 7549

(n,n') t

75-Re-193

Radionuclide Production Cross Section



18

Incident Energy (MeV)

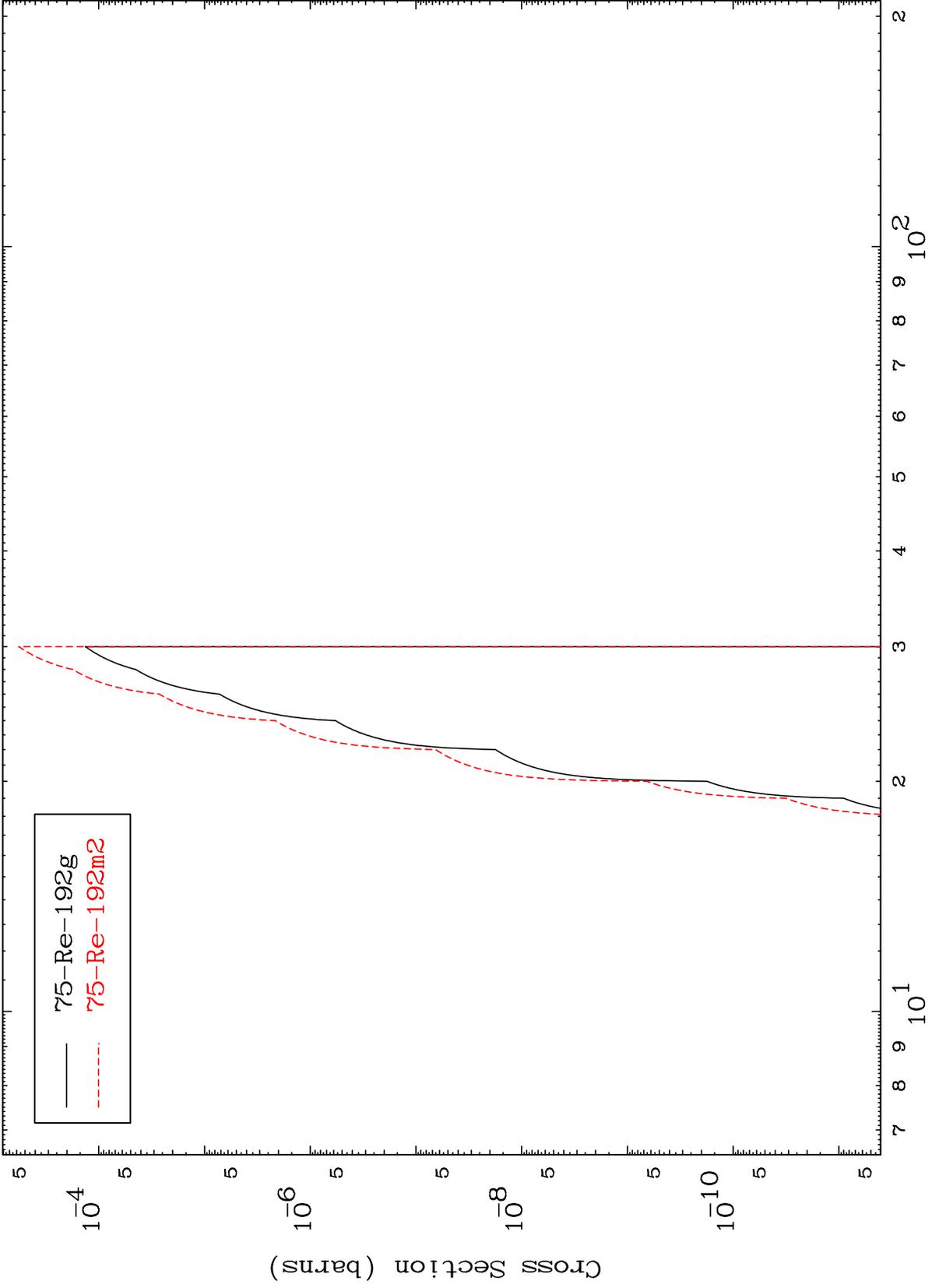
75-Re-193

MAT 7549

(n,n') He-3

75-Re-193

Radionuclide Production Cross Section



19

Incident Energy (MeV)

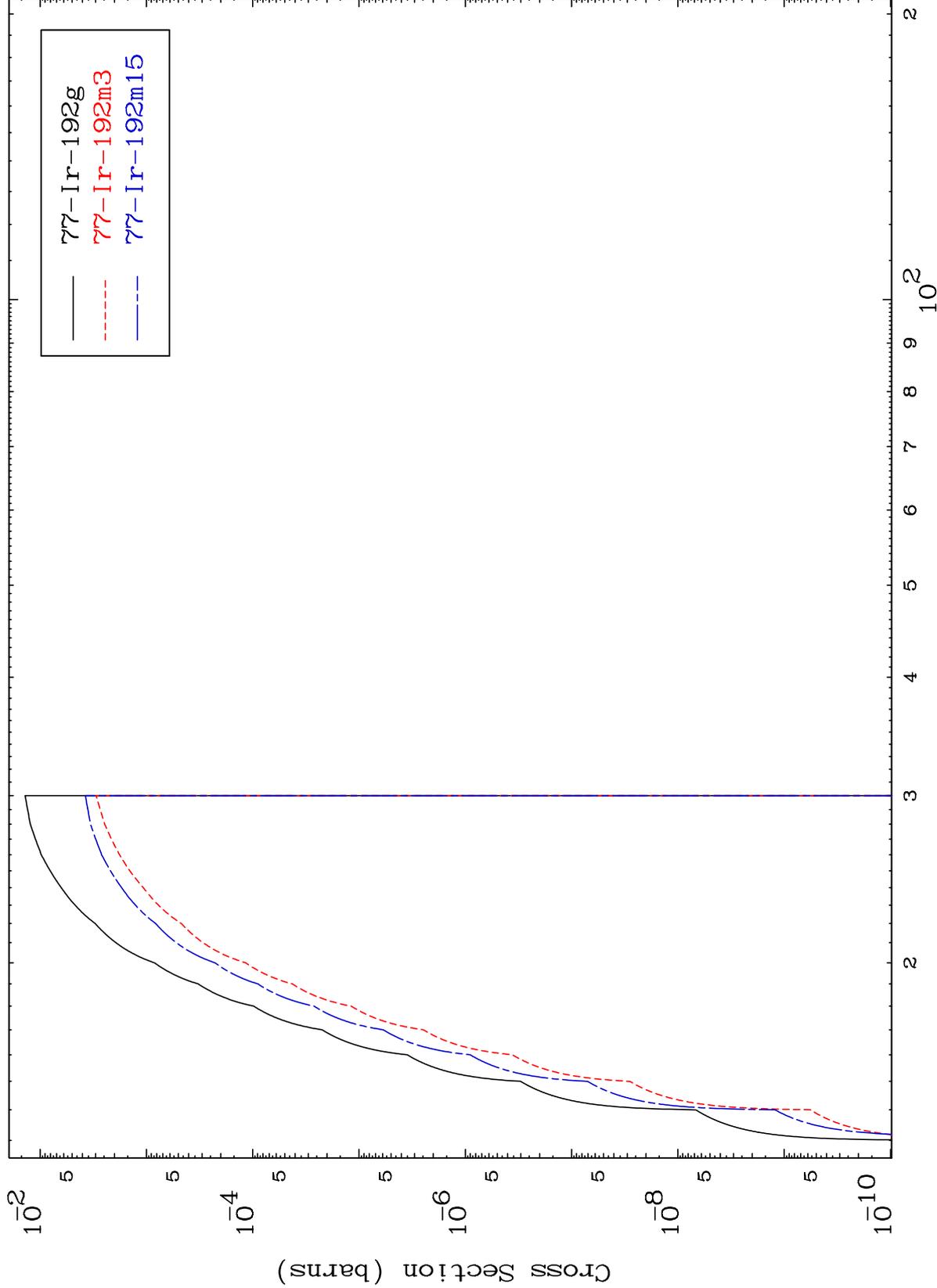
75-Re-193

MAT 7549

(n,4n)

75-Re-193

Radionuclide Production Cross Section



20

Incident Energy (MeV)

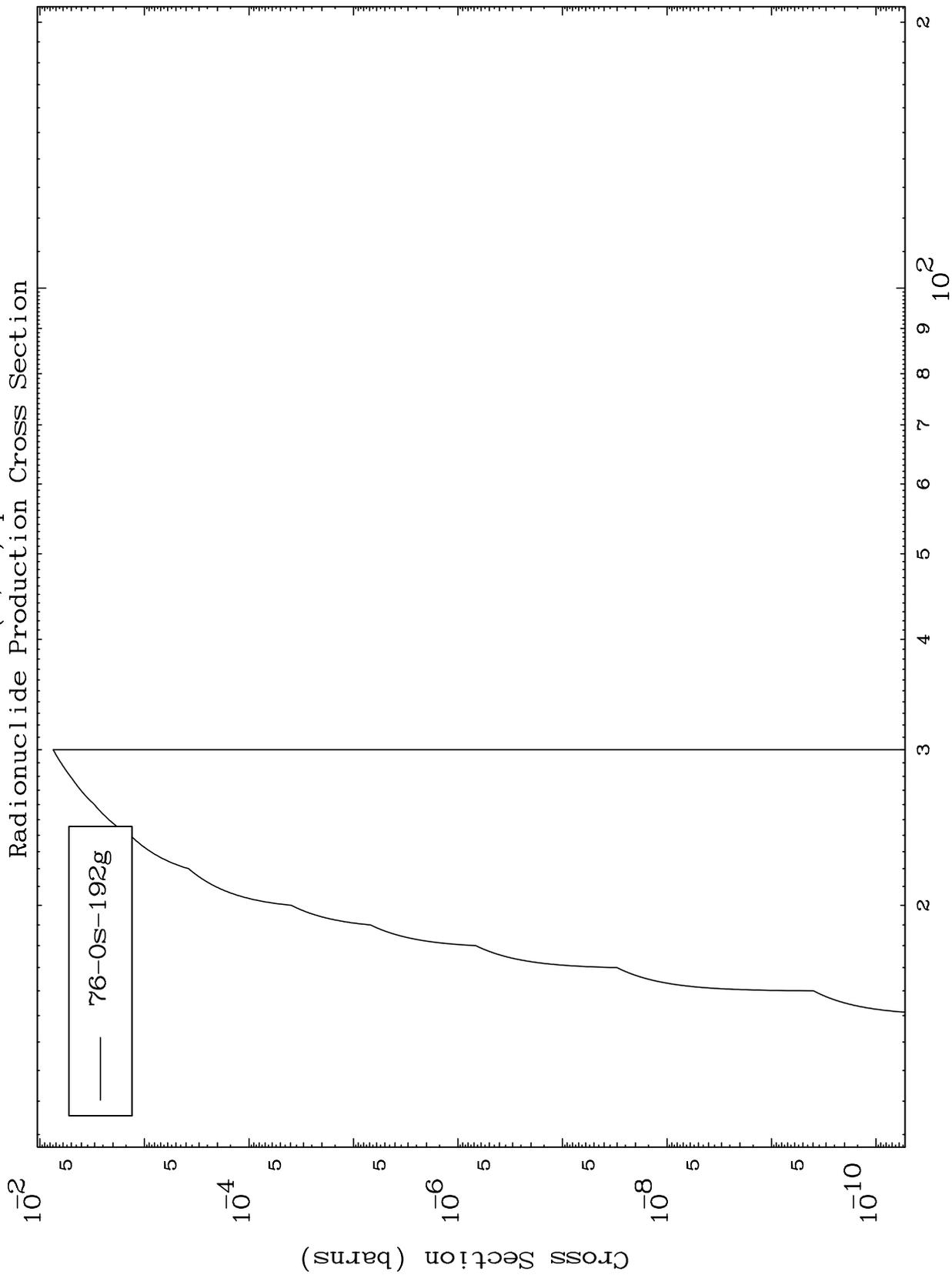
75-Re-193

MAT 7549

(n,3n) p

75-Re-193

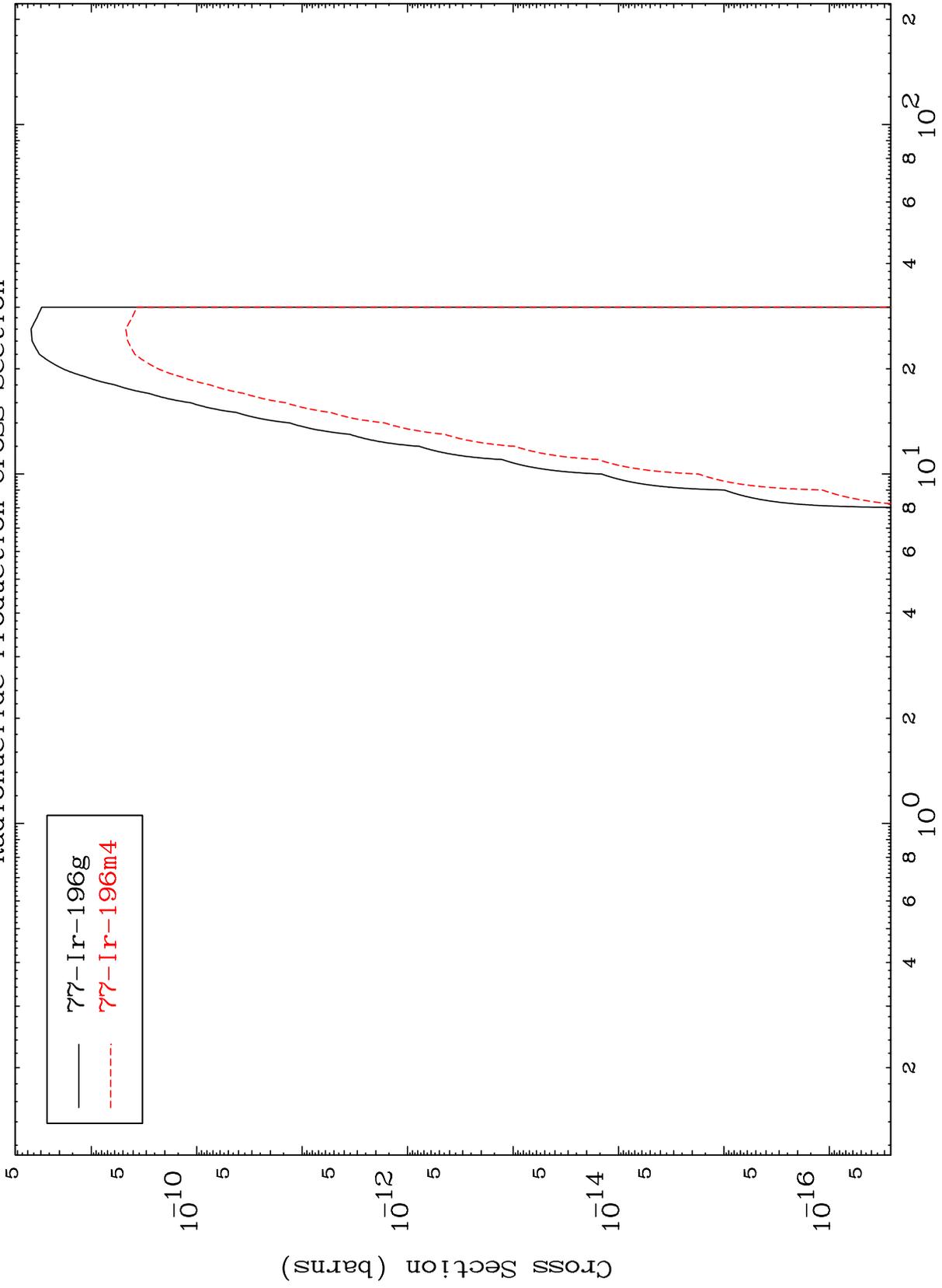
Radionuclide Production Cross Section



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75-Re-193

(n,  $\gamma$ )  
Radionuclide Production Cross Section

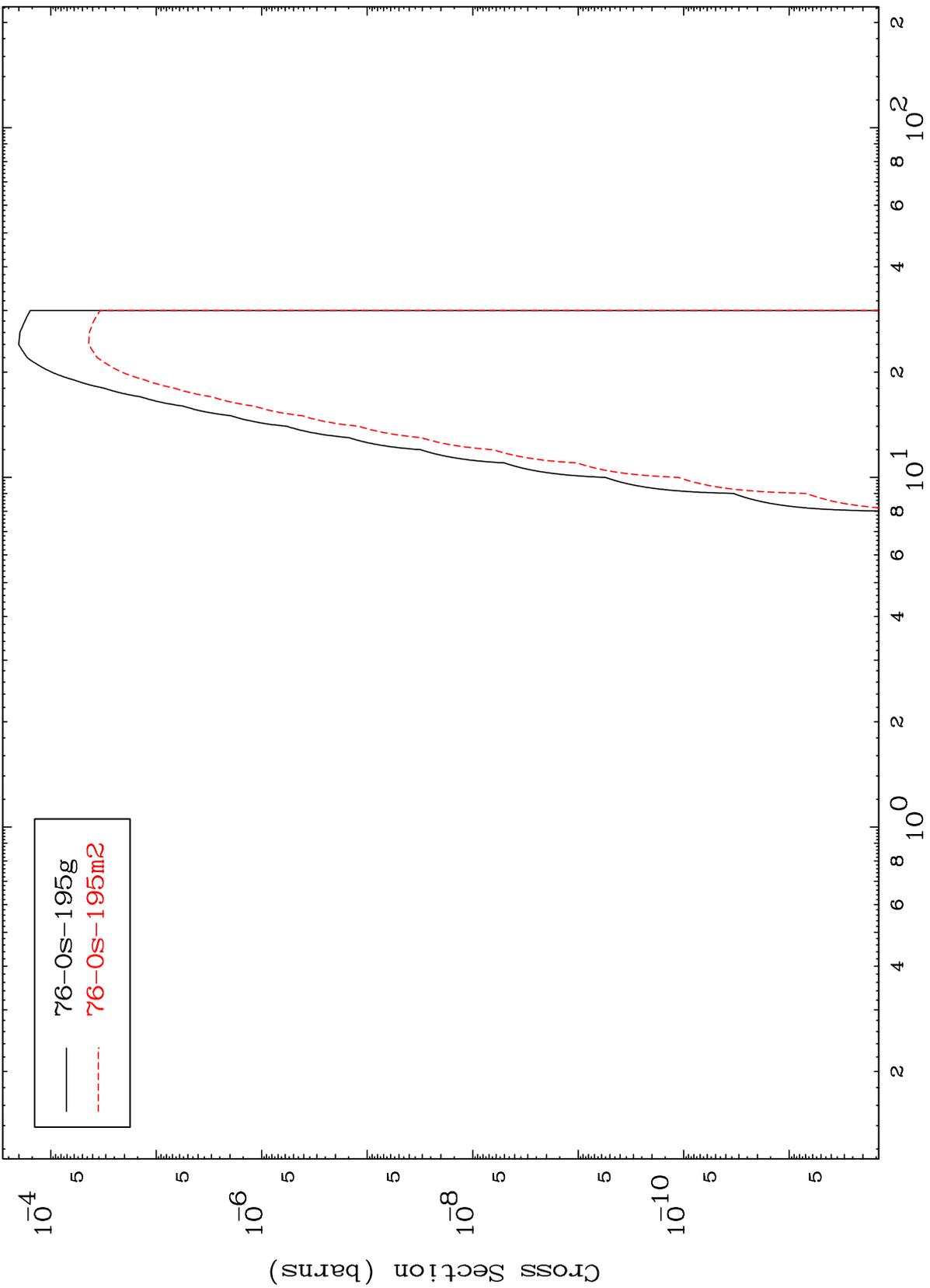


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(n,p)

75-Re-193

Radionuclide Production Cross Section

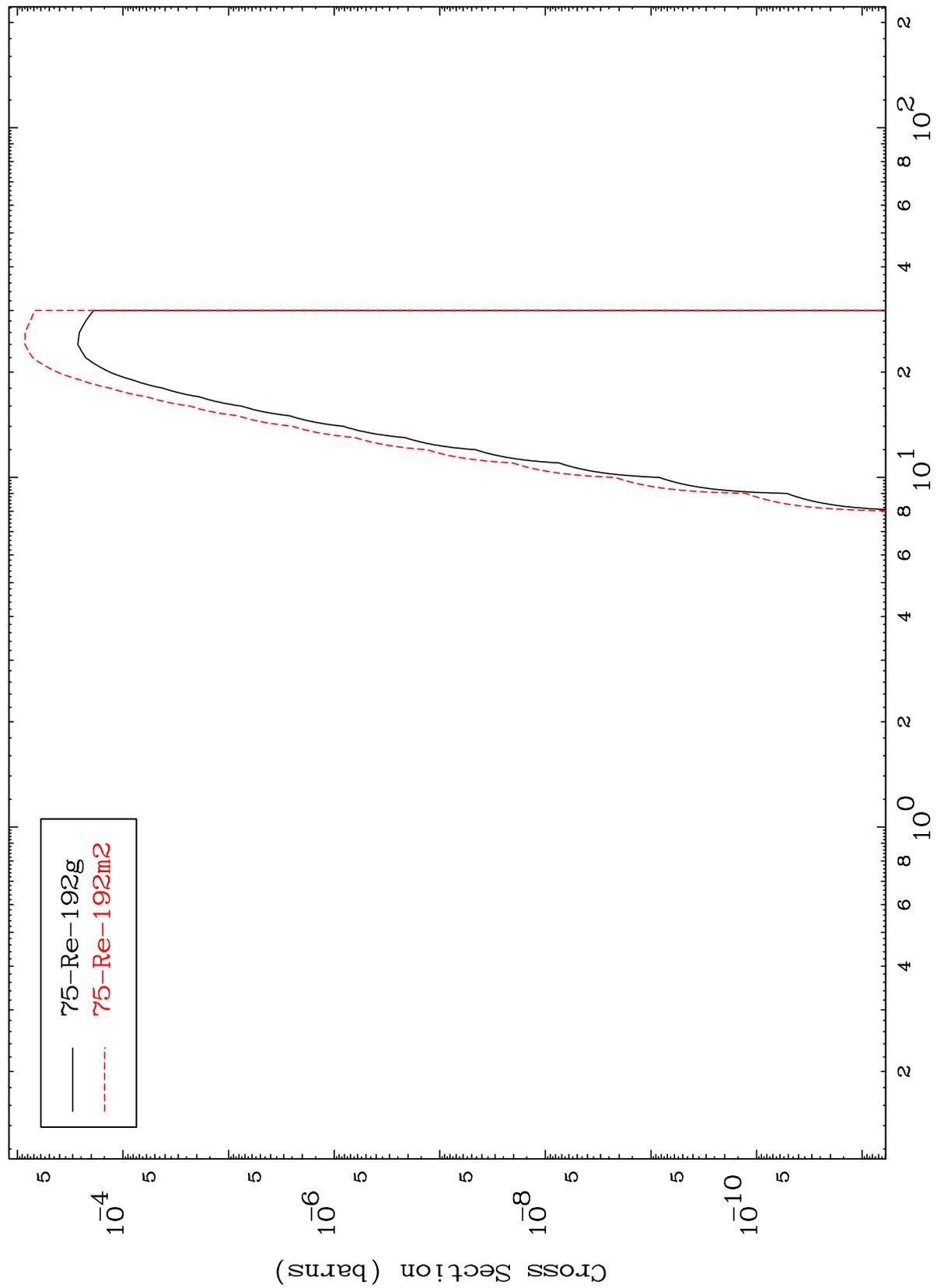


76-0s-195g  
76-0s-195m2

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75-Re-193

(n,  $\alpha$ )  
Radionuclide Production Cross Section



75-Re-193

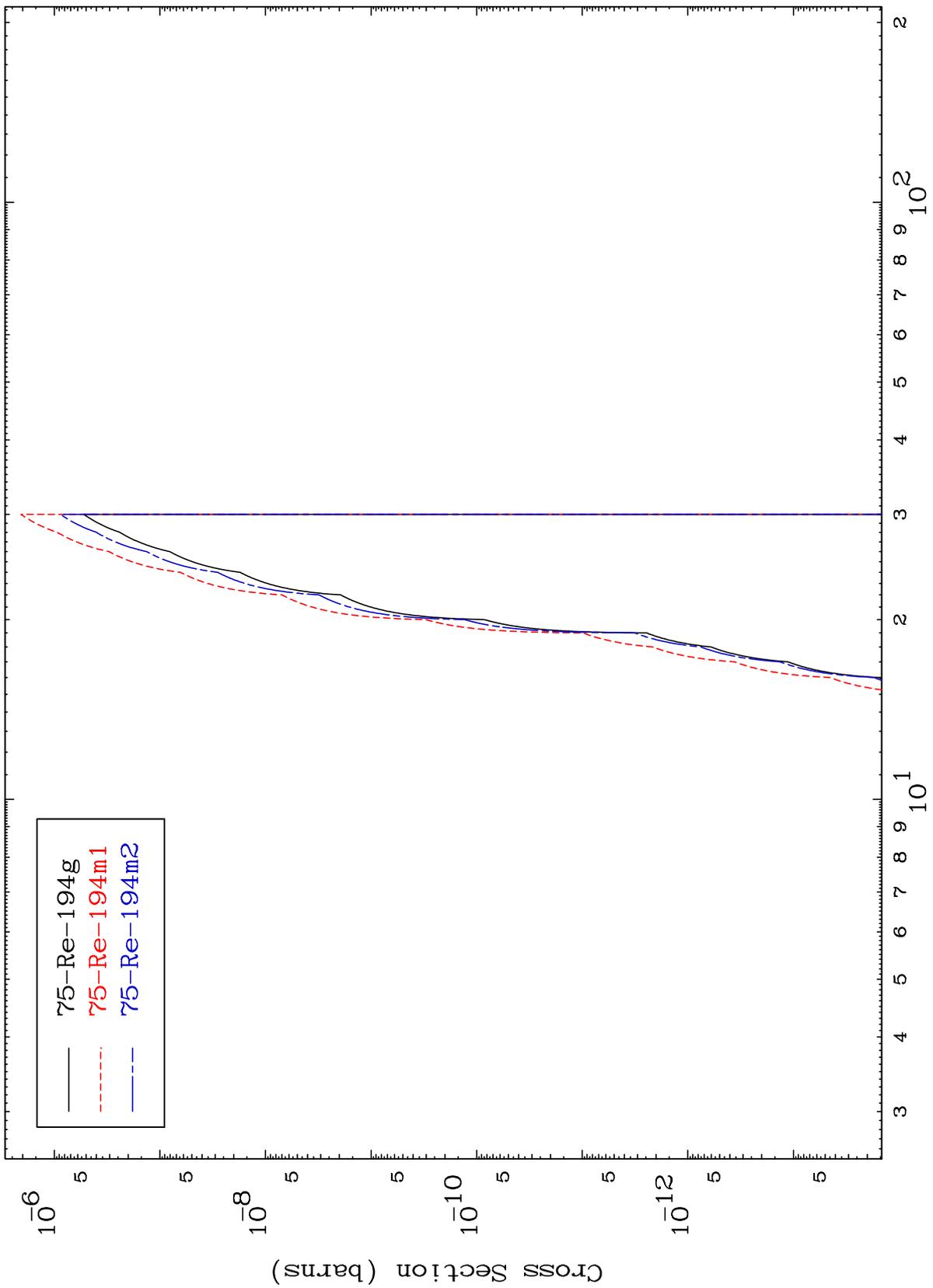
Incident Energy (MeV)

24

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75-Re-193

(n,2p)  
Radionuclide Production Cross Section



25

75-Re-193

Incident Energy (MeV)

Radionuclide Production Cross Section

