

Program EVALPLOT  
(Version 2021-1)

by

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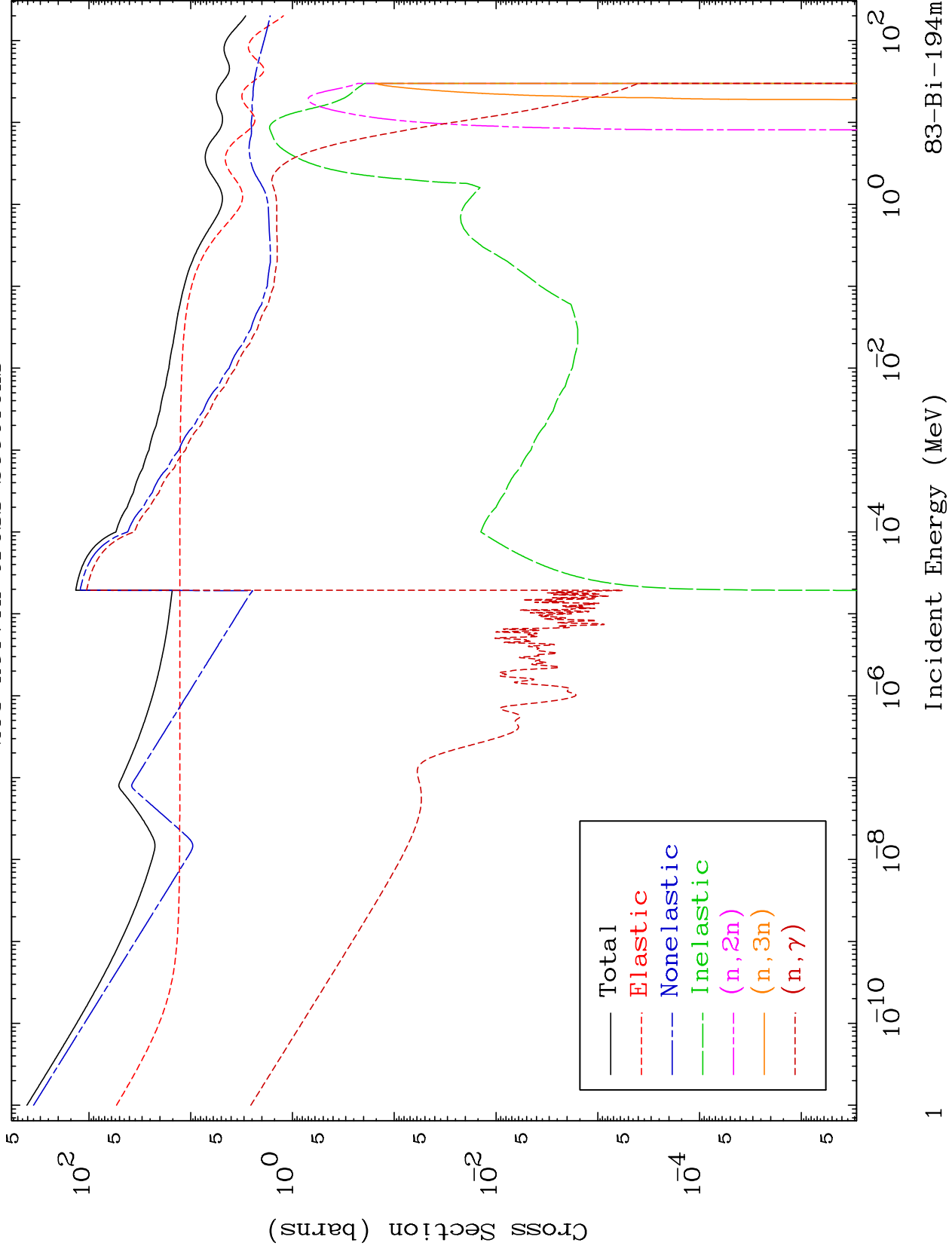
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8281

Neutron Major  
293 Kelvin Cross Sections

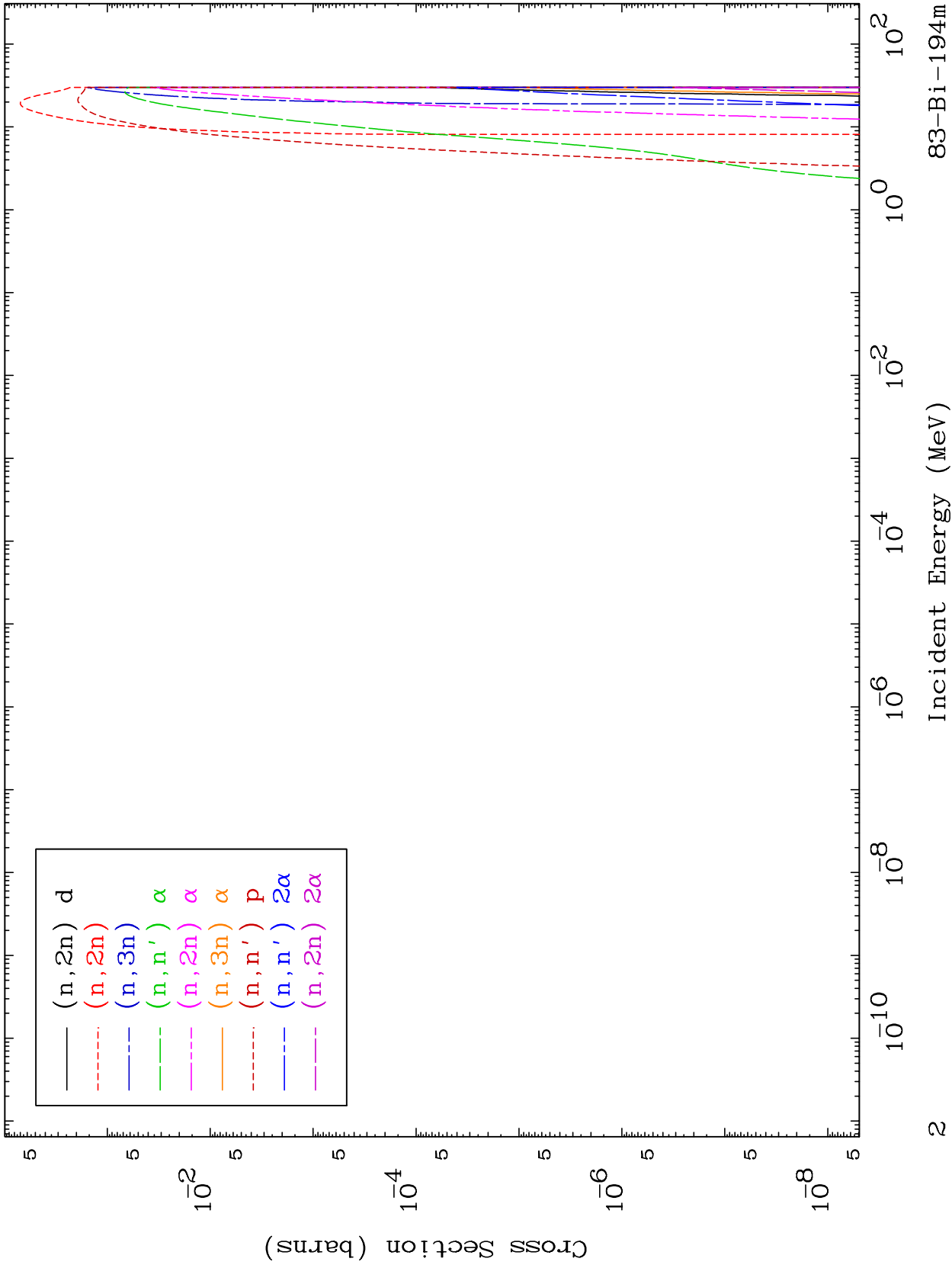
83-Bi-194m



MAT 8281

Neutron Absorption  
293 Kelvin Cross Sections

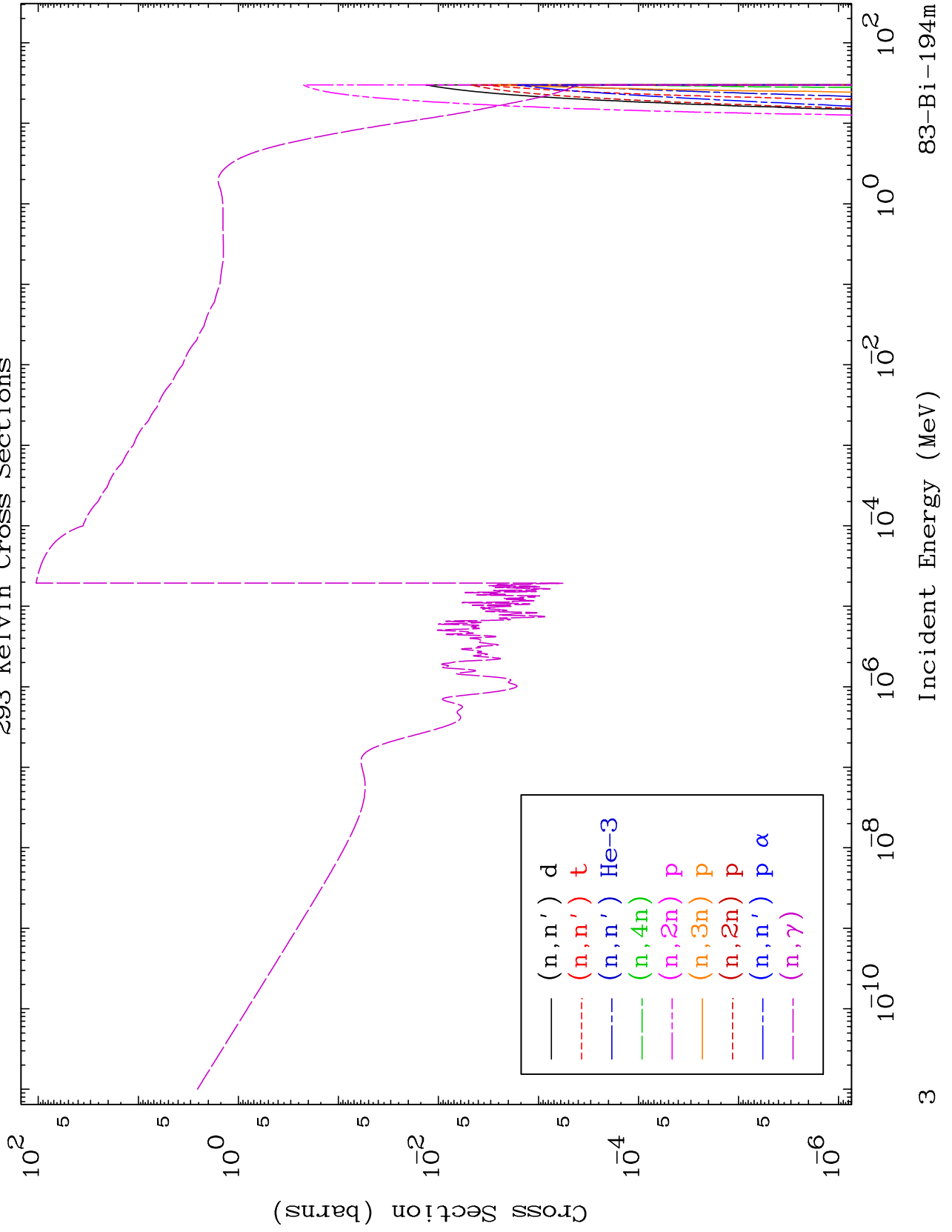
83-Bi-194m



MAT 8281

Neutron Absorption  
293 Kelvin Cross Sections

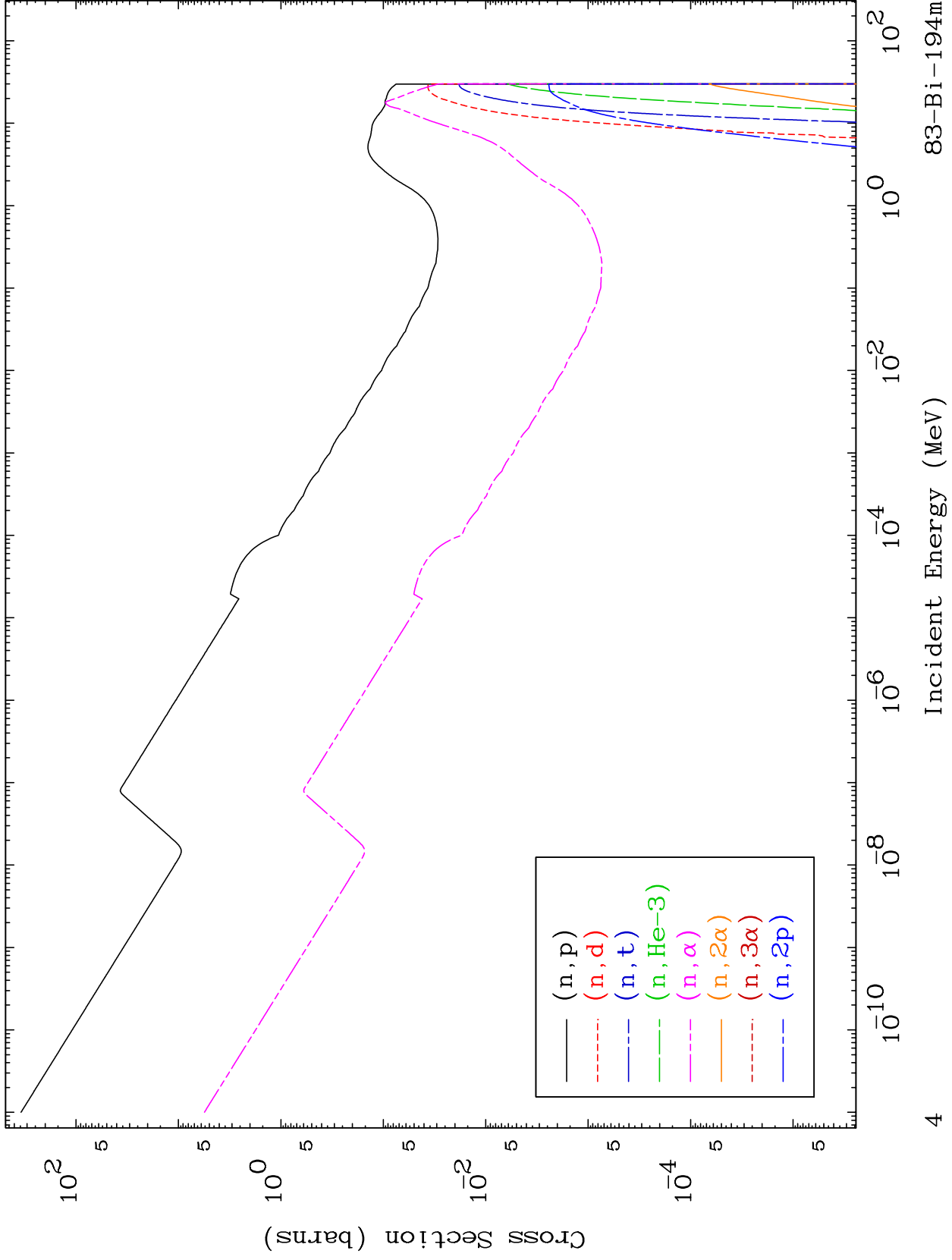
83-Bi-194m



MAT 8281

Neutron Absorption  
293 Kelvin Cross Sections

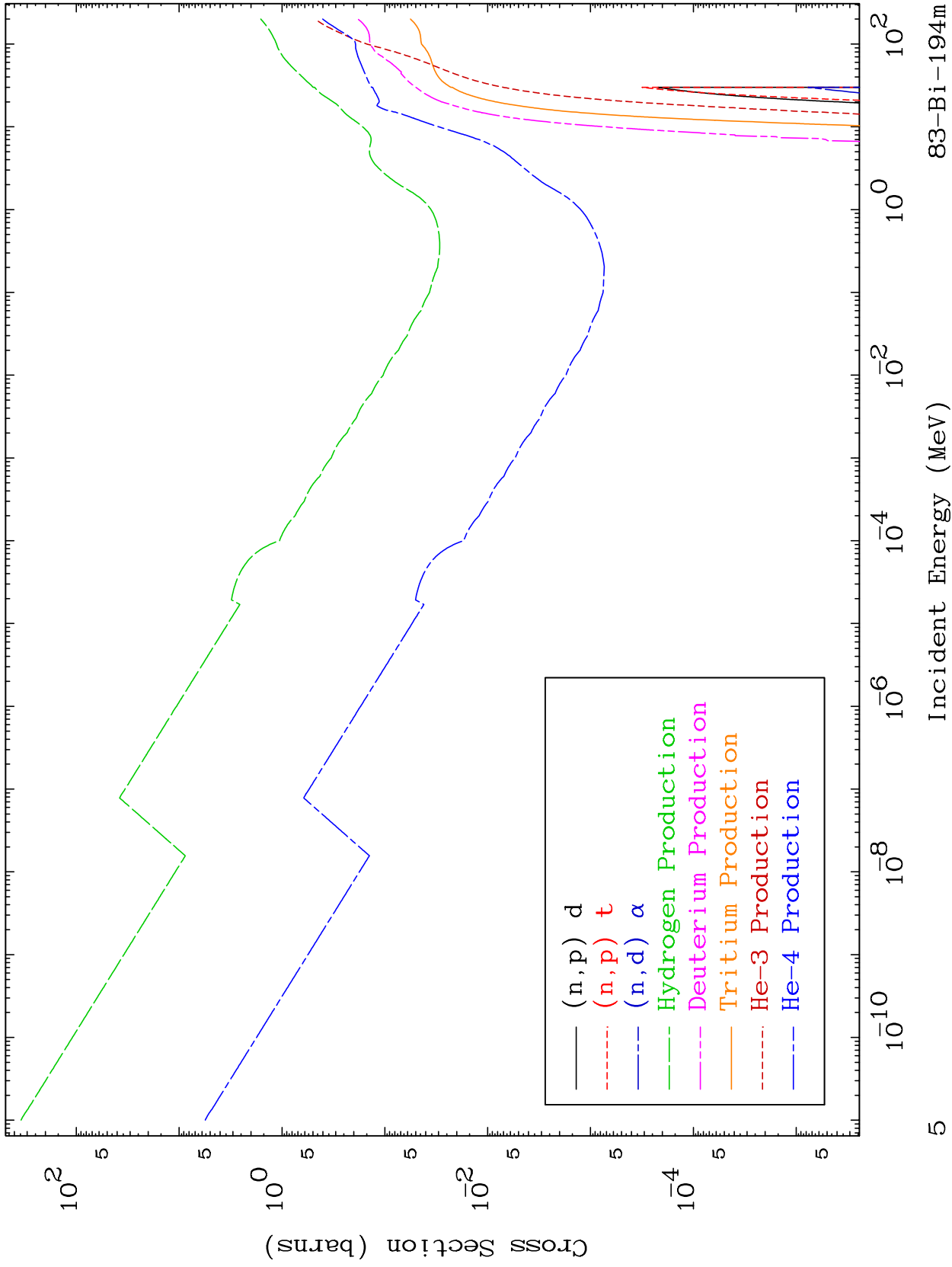
83-Bi-194m



MAT 8281

Neutron Absorption  
293 Kelvin Cross Sections

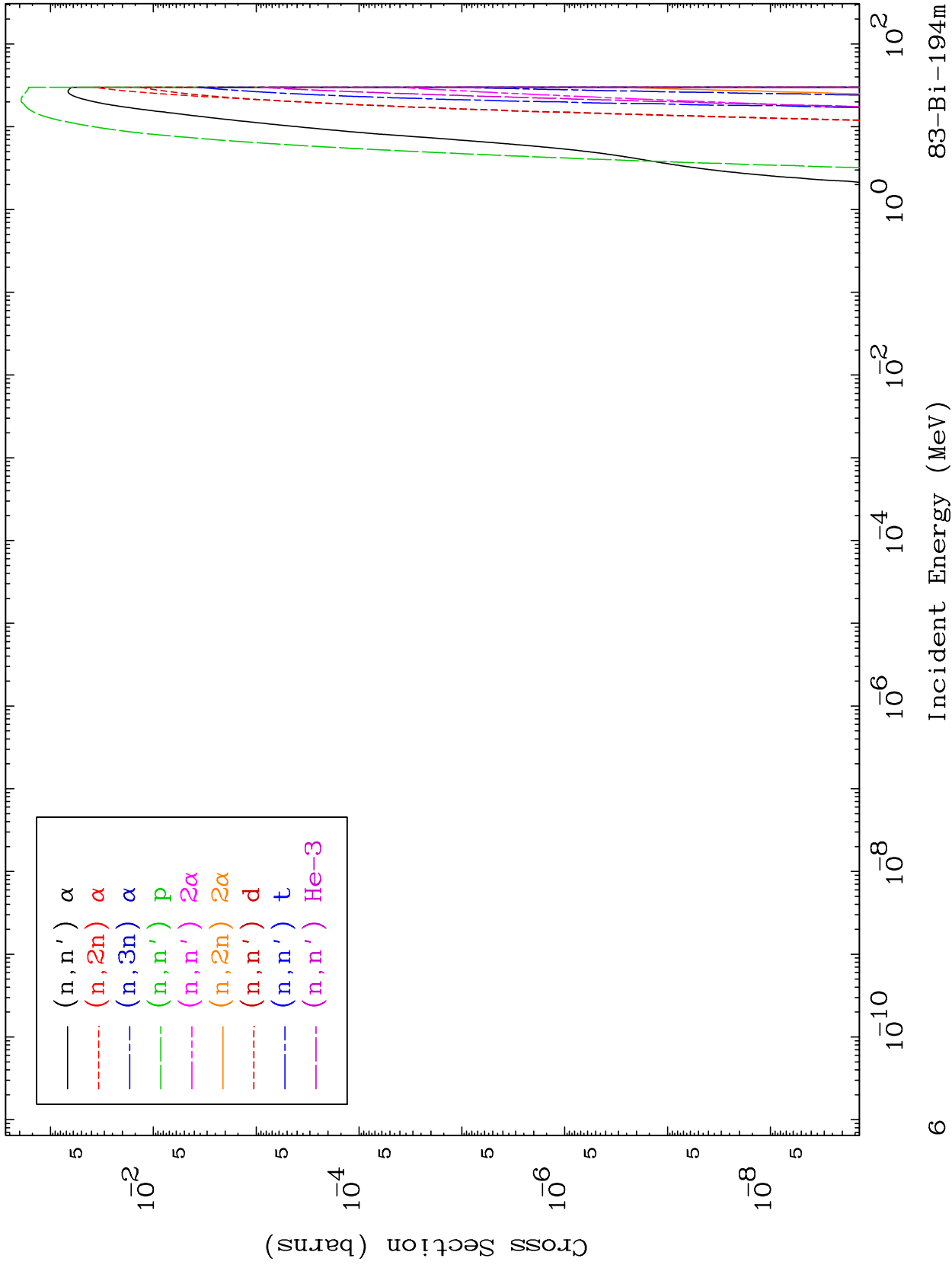
83-Bi-194m



MAT 8281

Charged Particle  
293 Kelvin Cross Sections

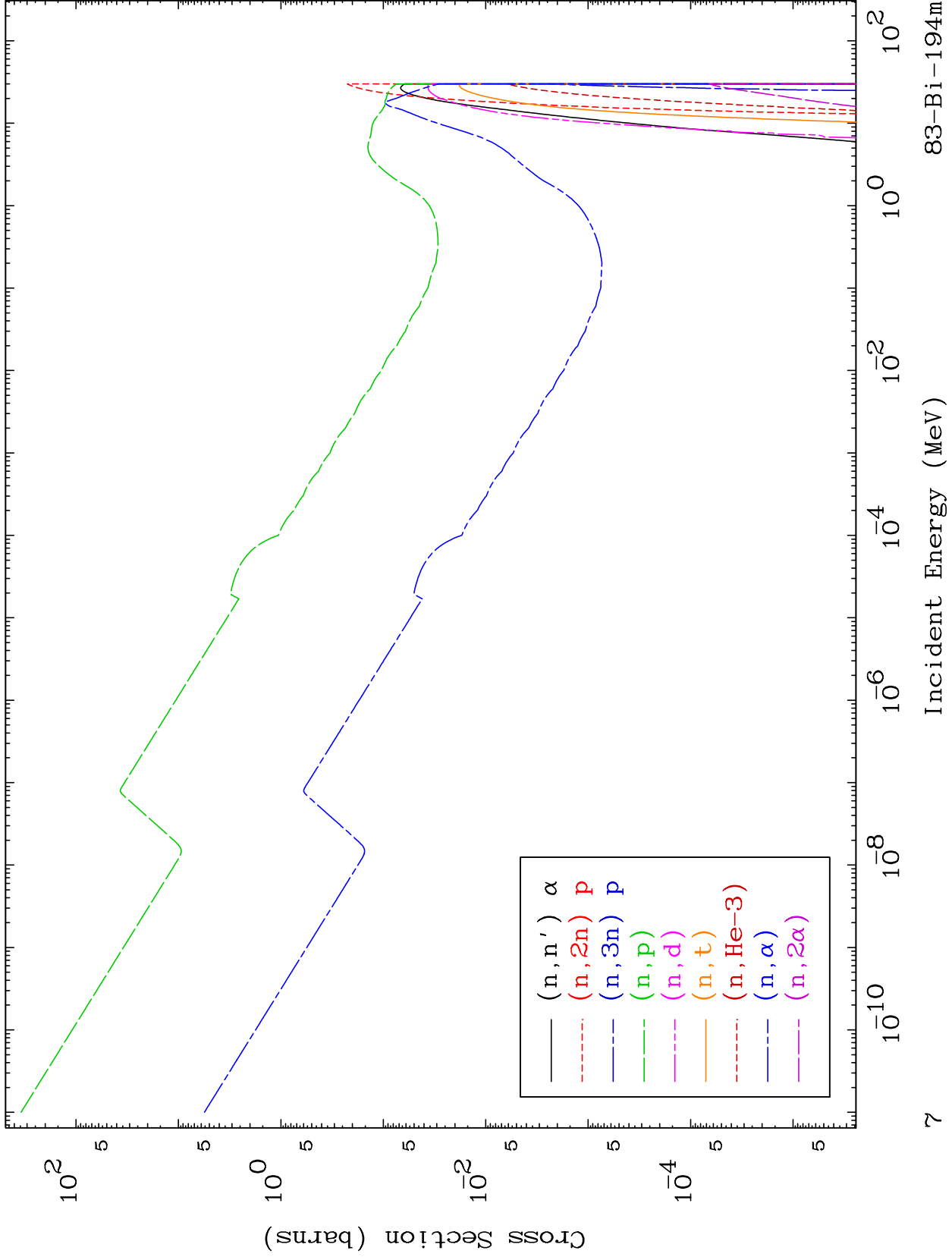
83-Bi-194m



MAT 8281

Charged Particle  
293 Kelvin Cross Sections

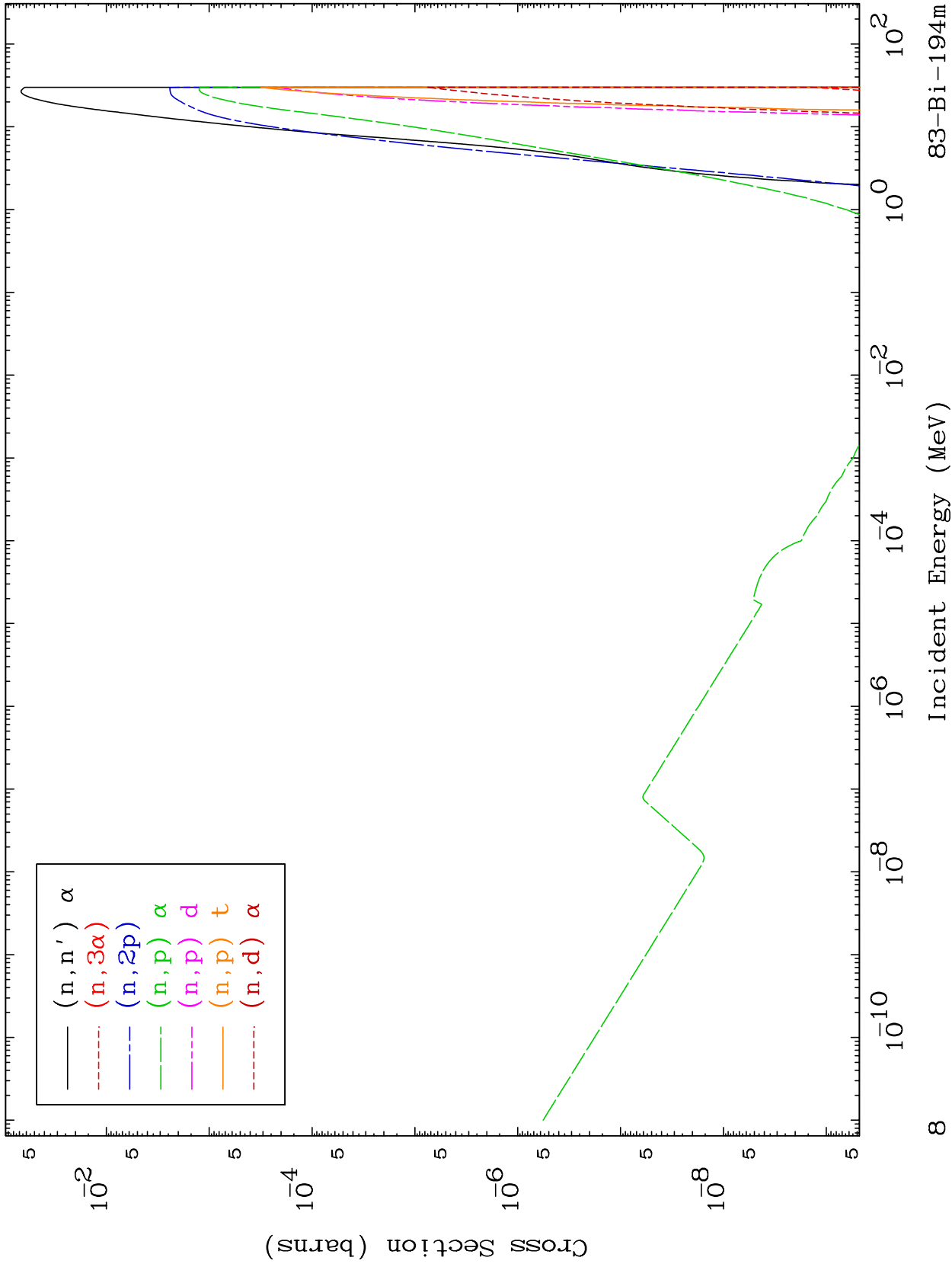
83-Bi-194m



MAT 8281

Charged Particle  
293 Kelvin Cross Sections

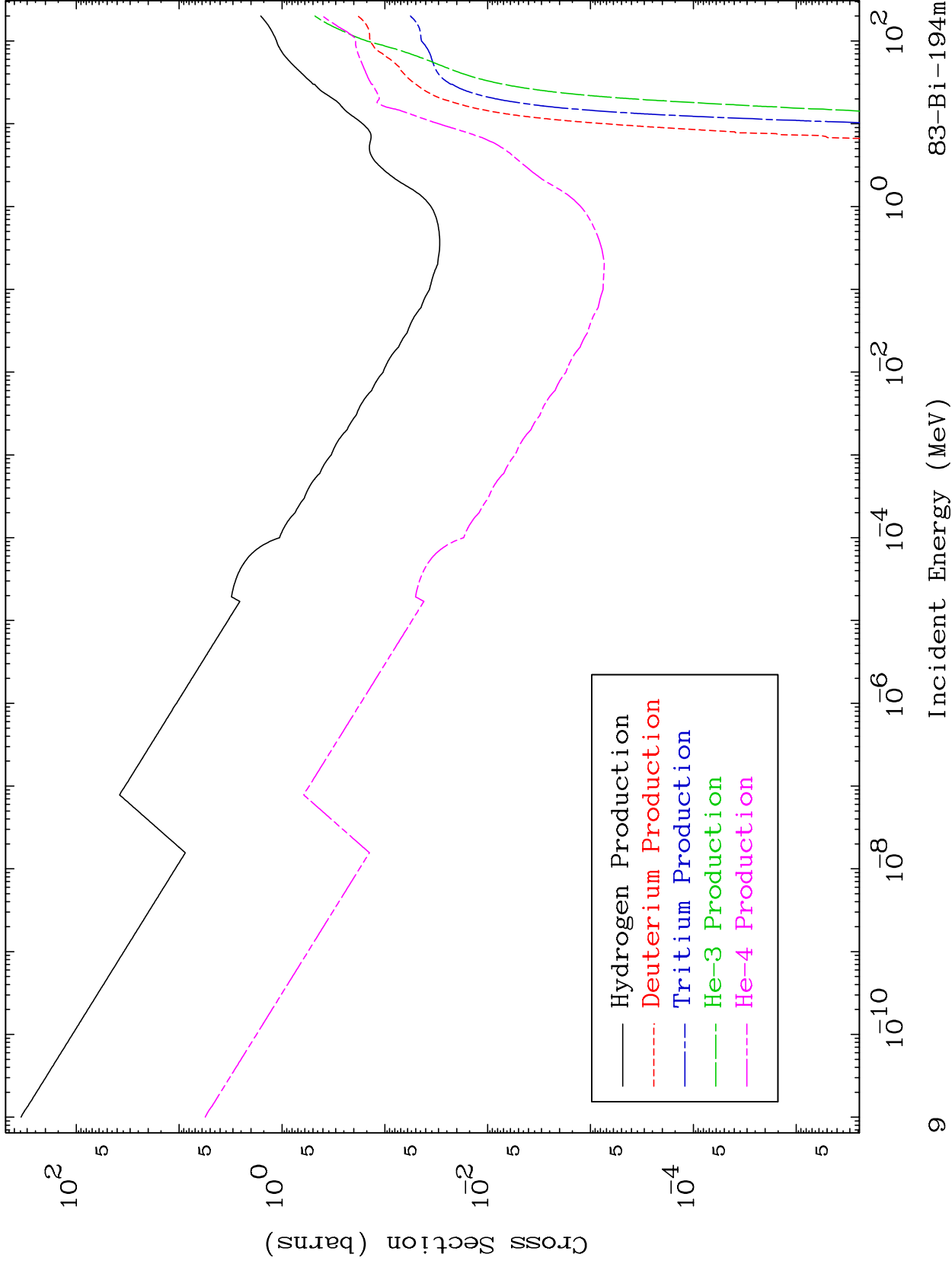
83-Bi-194m



MAT 8281

Particle Production  
293 Kelvin Cross Sections

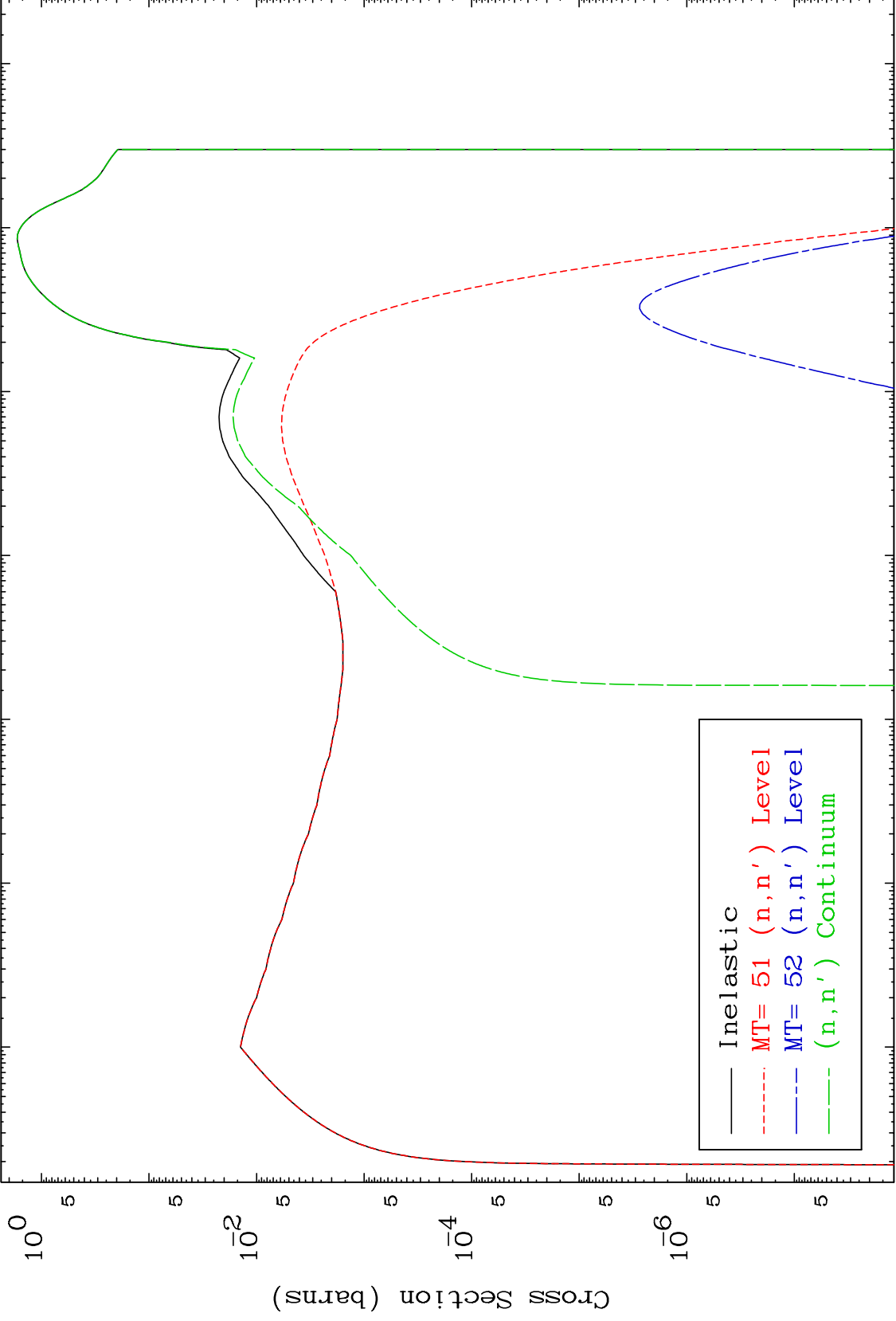
83-Bi-194m



MAT 8281

(n,n') Levels  
293 Kelvin Cross Sections

83-Bi-194m



10

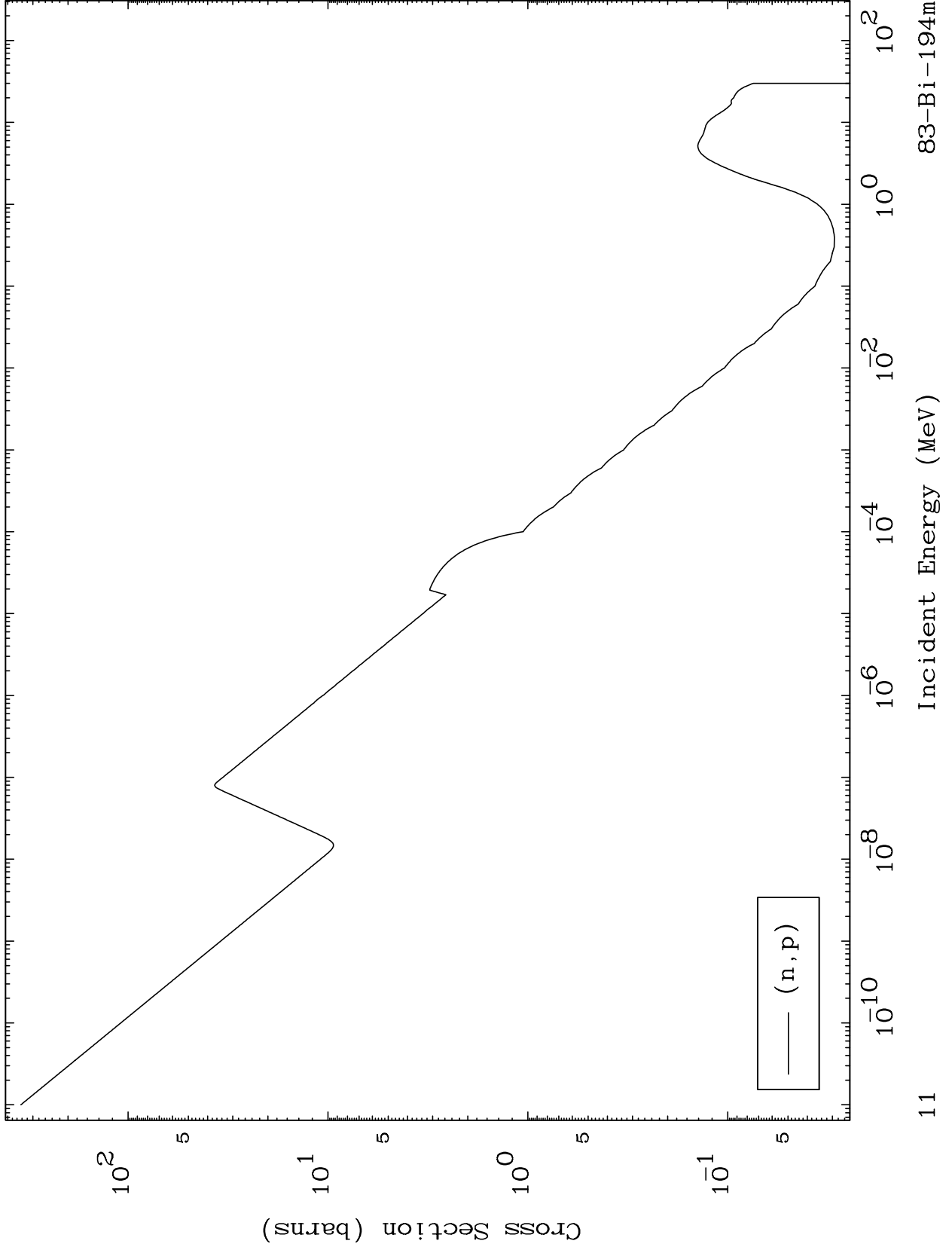
Incident Energy (MeV)

83-Bi-194m

MAT 8281

(n,p) Levels  
293 Kelvin Cross Sections

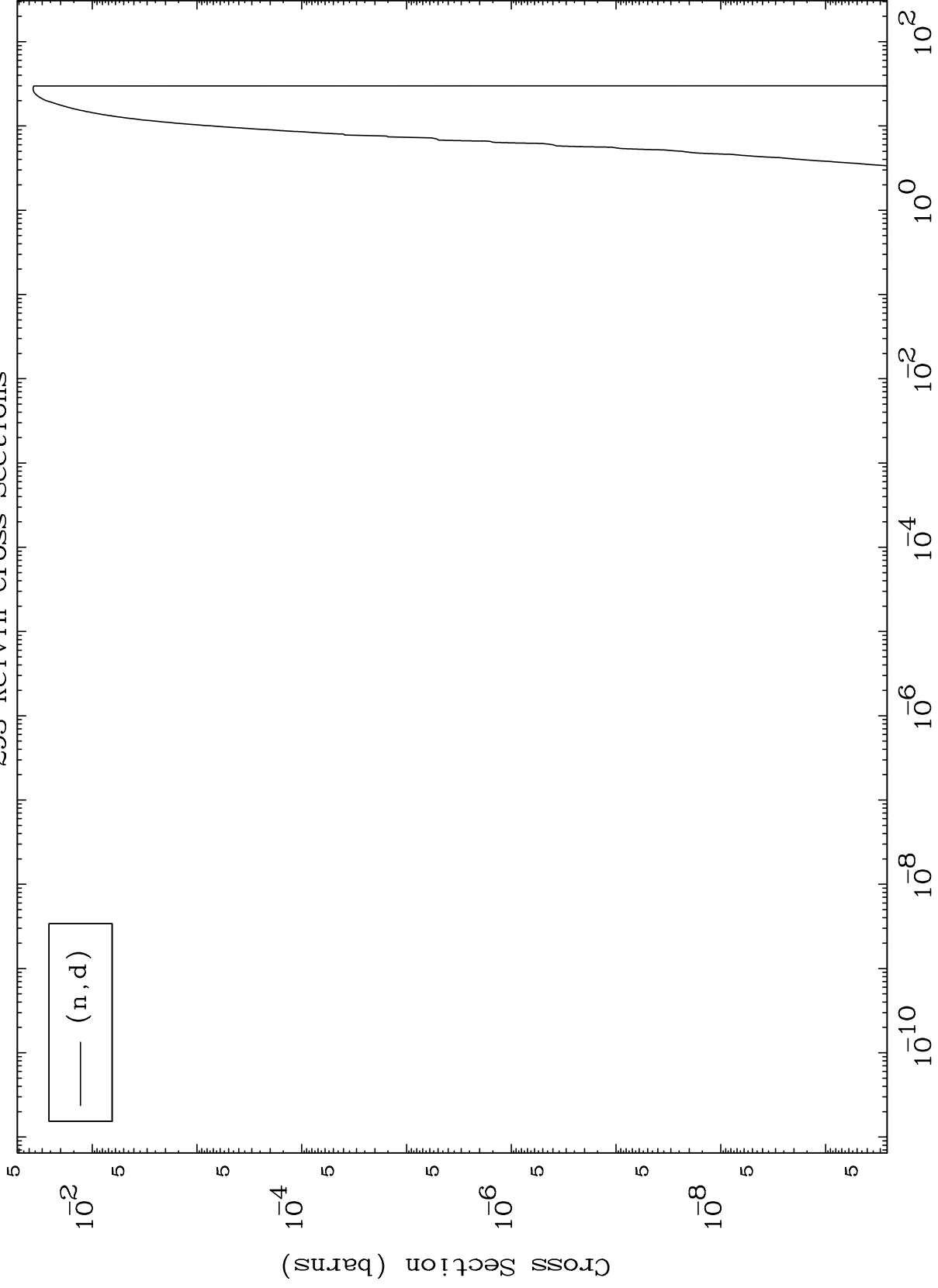
83-Bi-194m



MAT 8281

(n,d) Levels  
293 Kelvin Cross Sections

83-Bi-194m



12

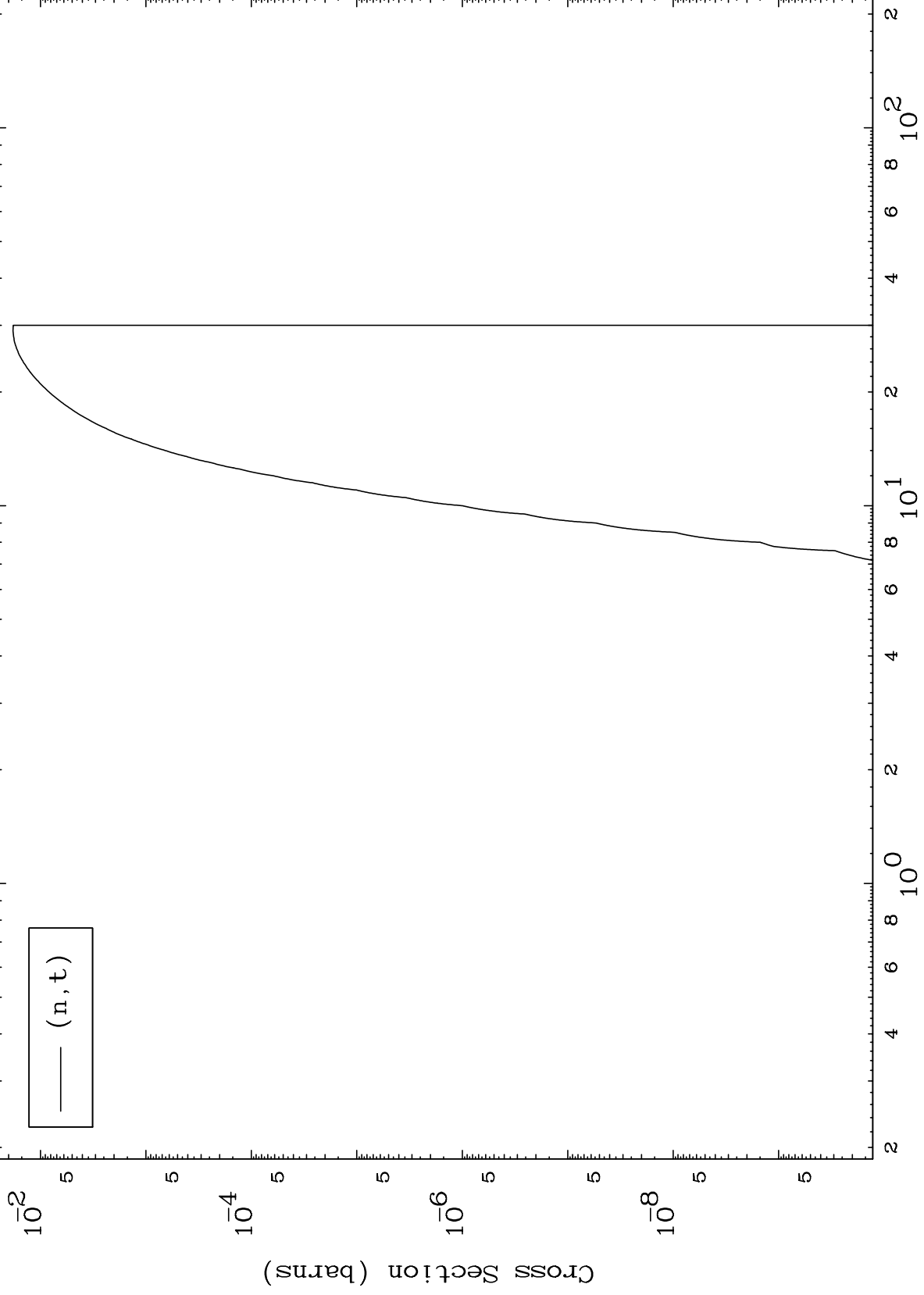
Incident Energy (MeV)

83-Bi-194m

MAT 8281

(n,t) Levels  
293 Kelvin Cross Sections

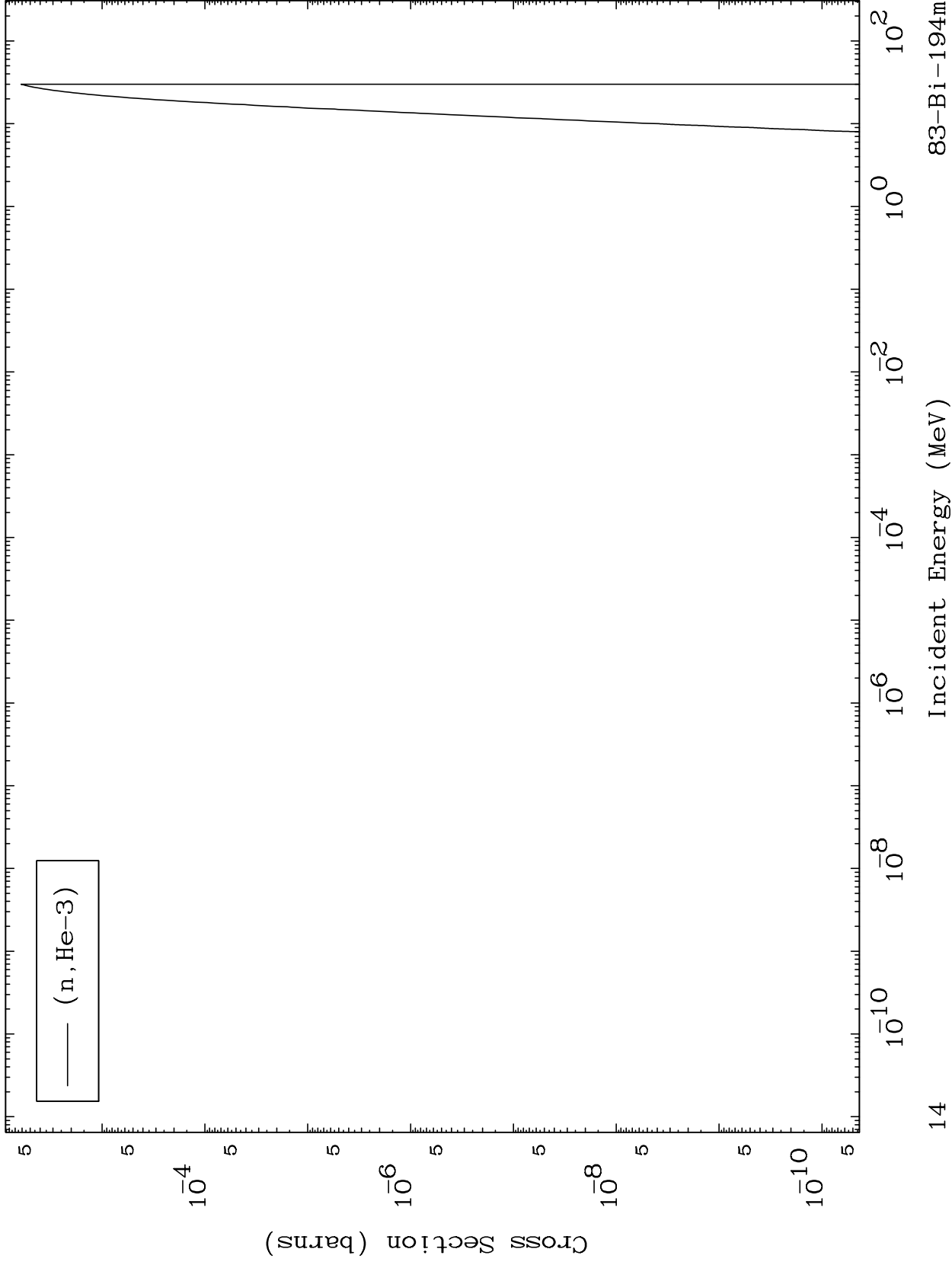
83-Bi-194m



MAT 8281

(n,He3) Levels  
293 Kelvin Cross Sections

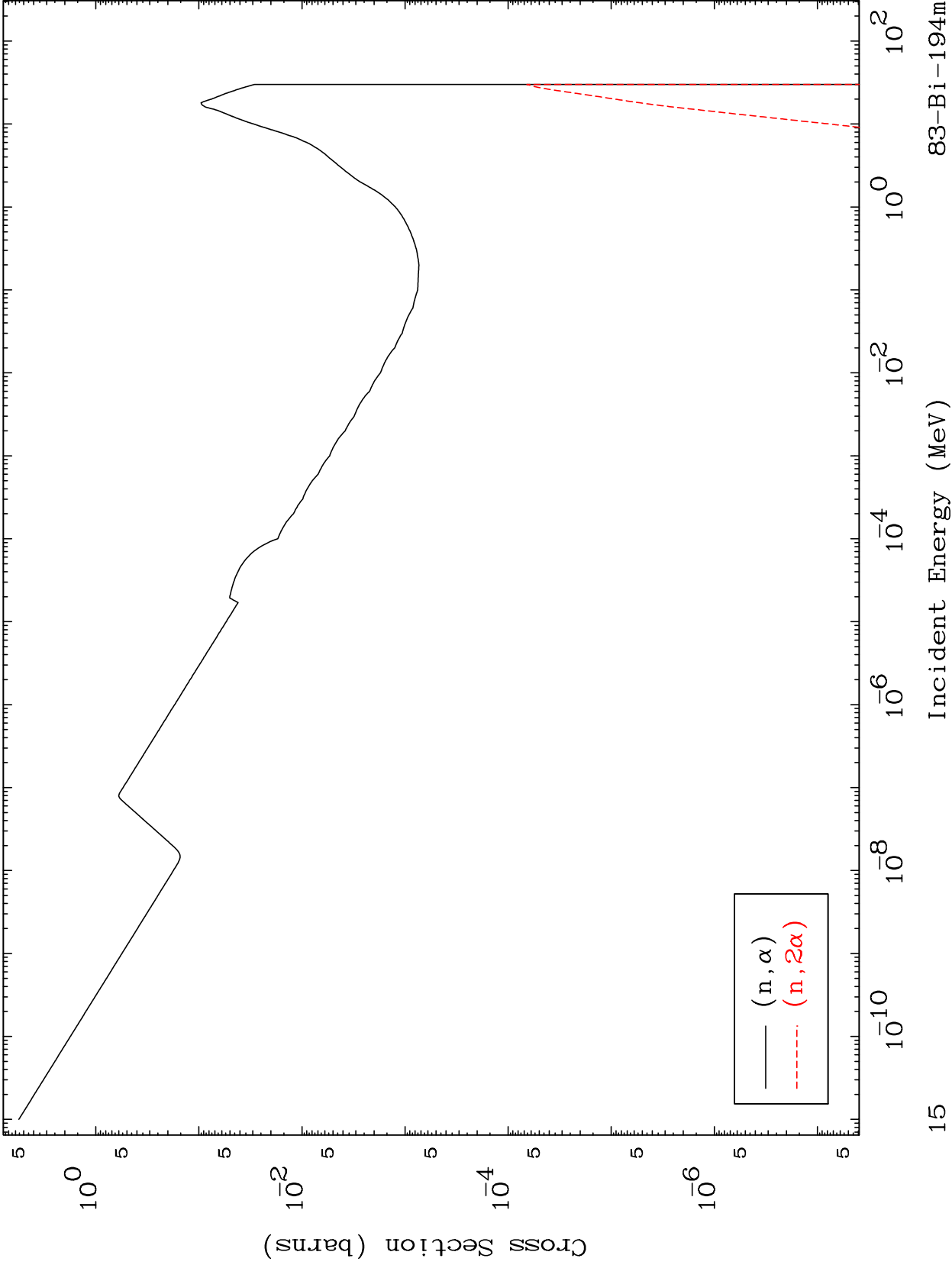
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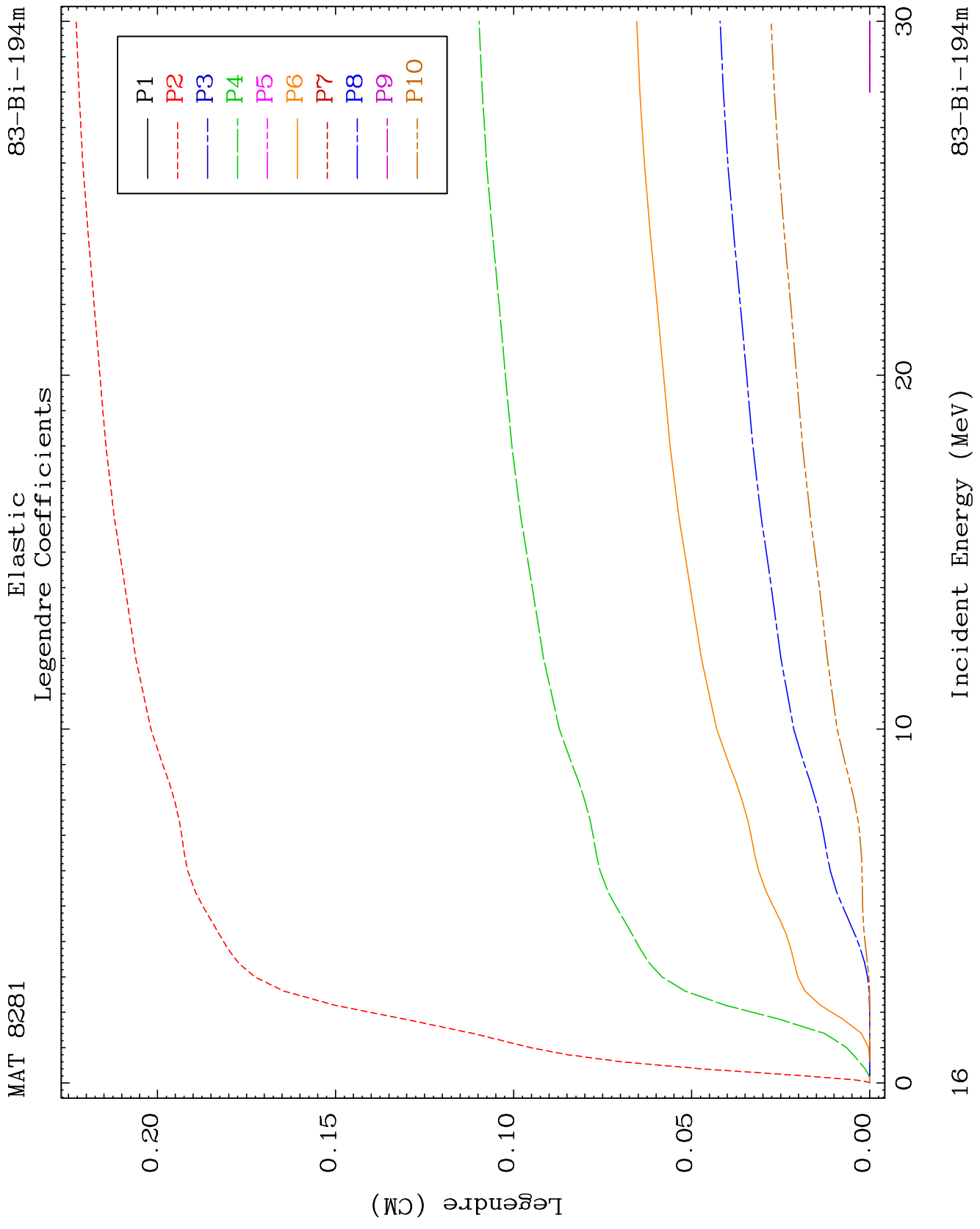


MAT 8281

(n,α) Levels  
293 Kelvin Cross Sections

83-Bi-194m





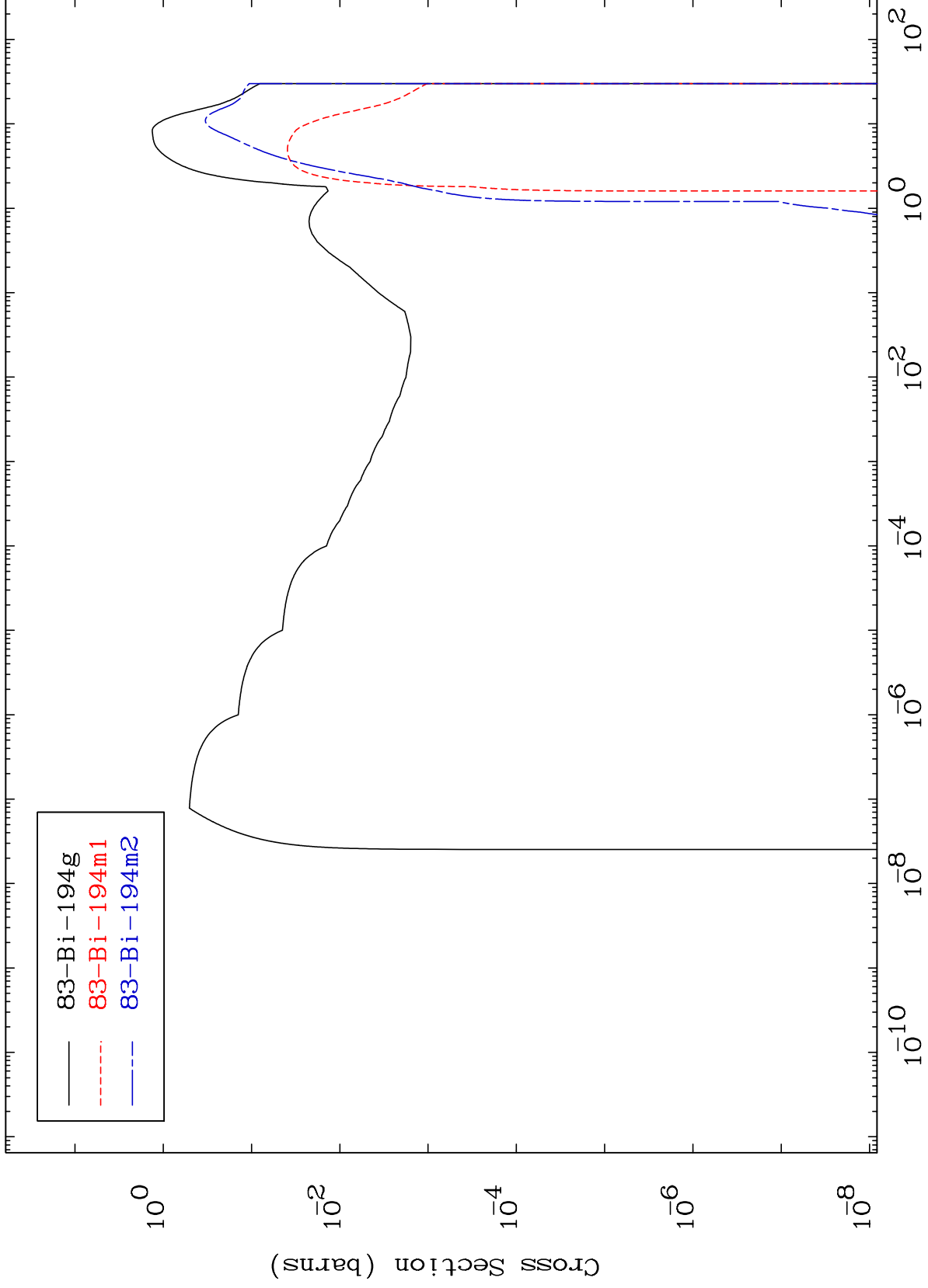




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Radionuclide Production Cross Section

$^{83}\text{Bi}-194\text{m}$



19

Incident Energy (MeV)

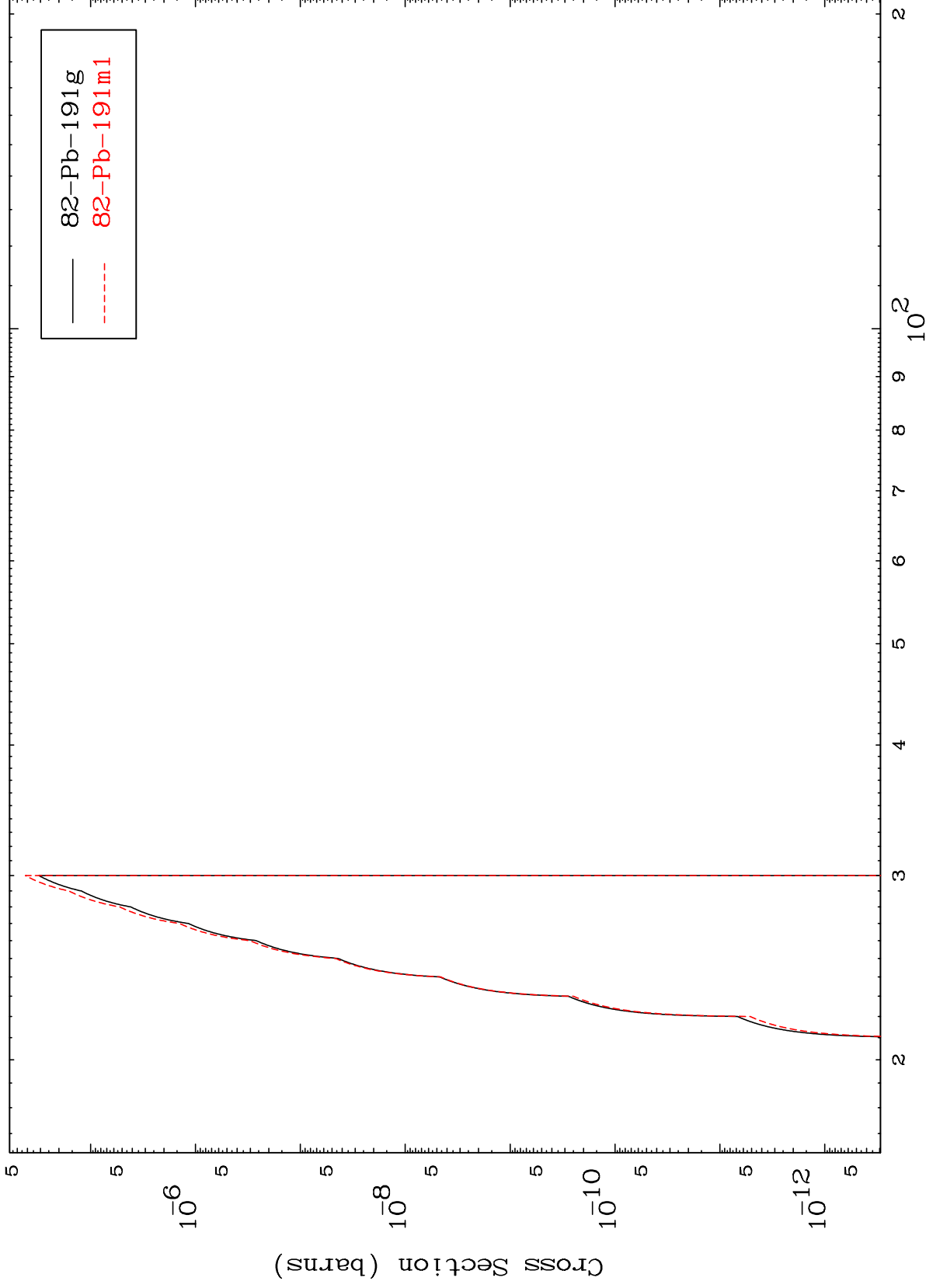
$^{83}\text{Bi}-194\text{m}$

MAT 8281

(n,2n) d

83-Bi-194m

Radionuclide Production Cross Section



20

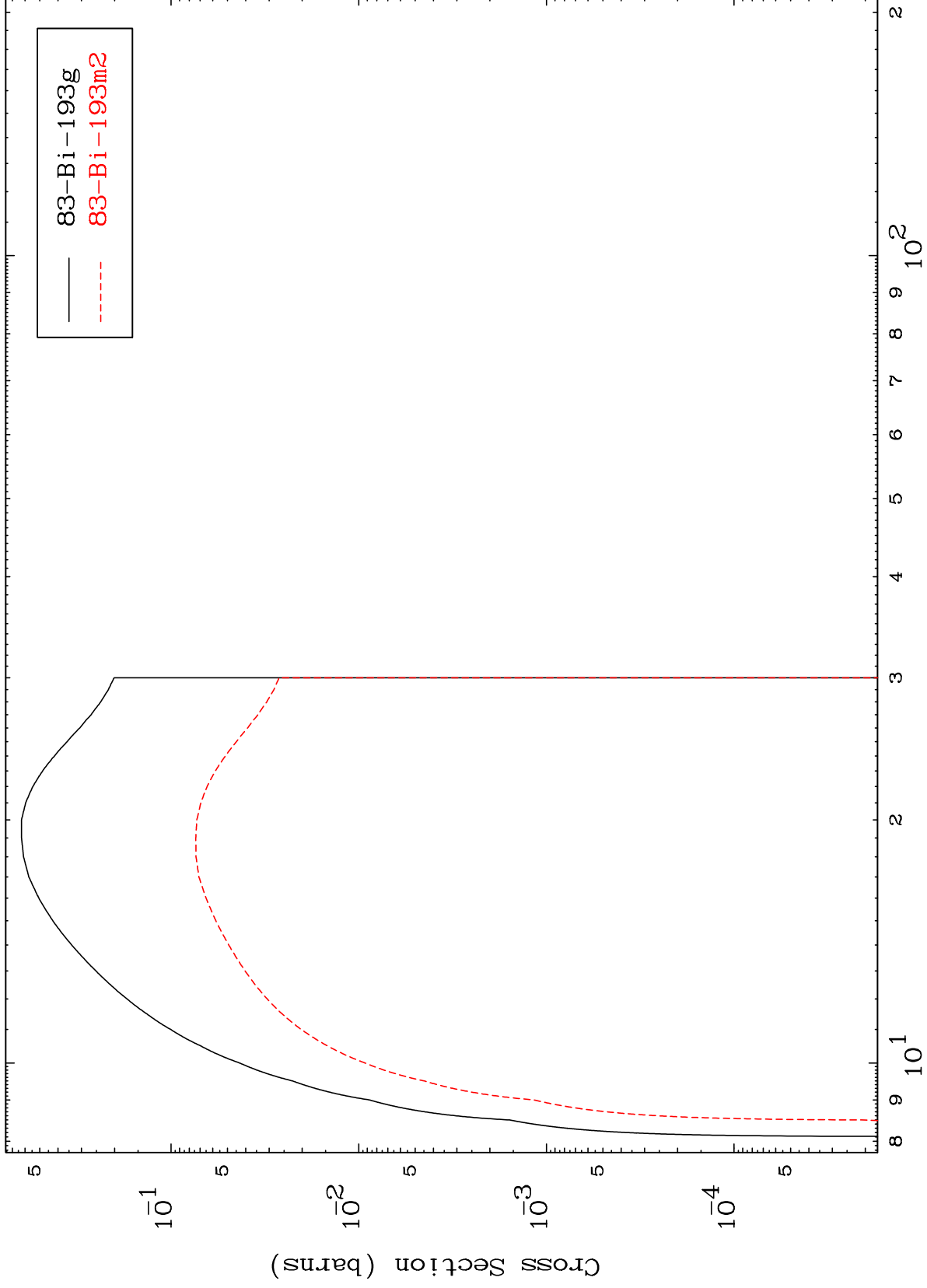
Incident Energy (MeV)

83-Bi-194m

MAT 8281

83-Bi-194m

(n,2n)  
Radionuclide Production Cross Section



83-Bi-194m

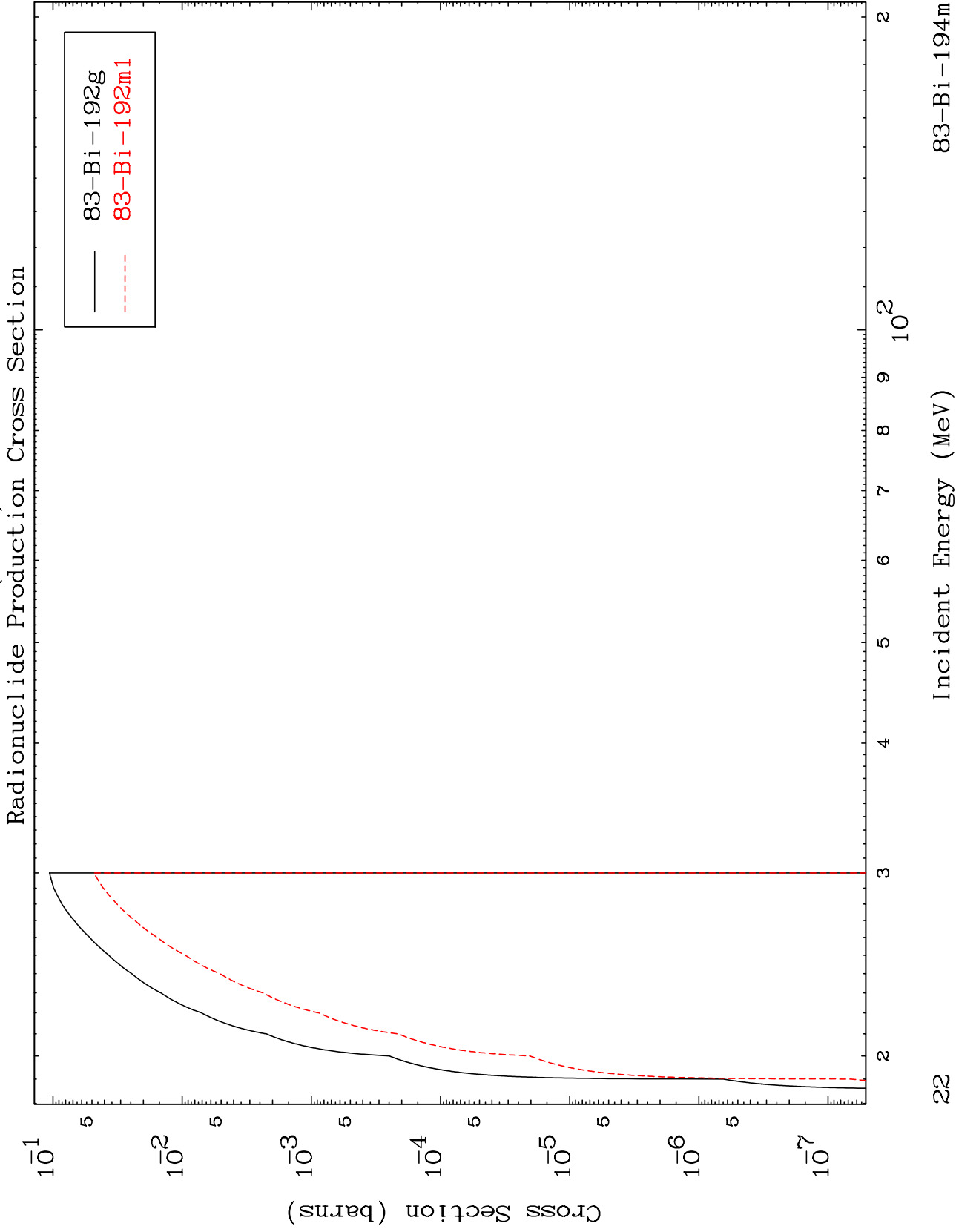
Incident Energy (MeV)

21

MAT 8281

(n,3n)

<sup>83</sup>Bi-194m



22

Incident Energy (MeV)

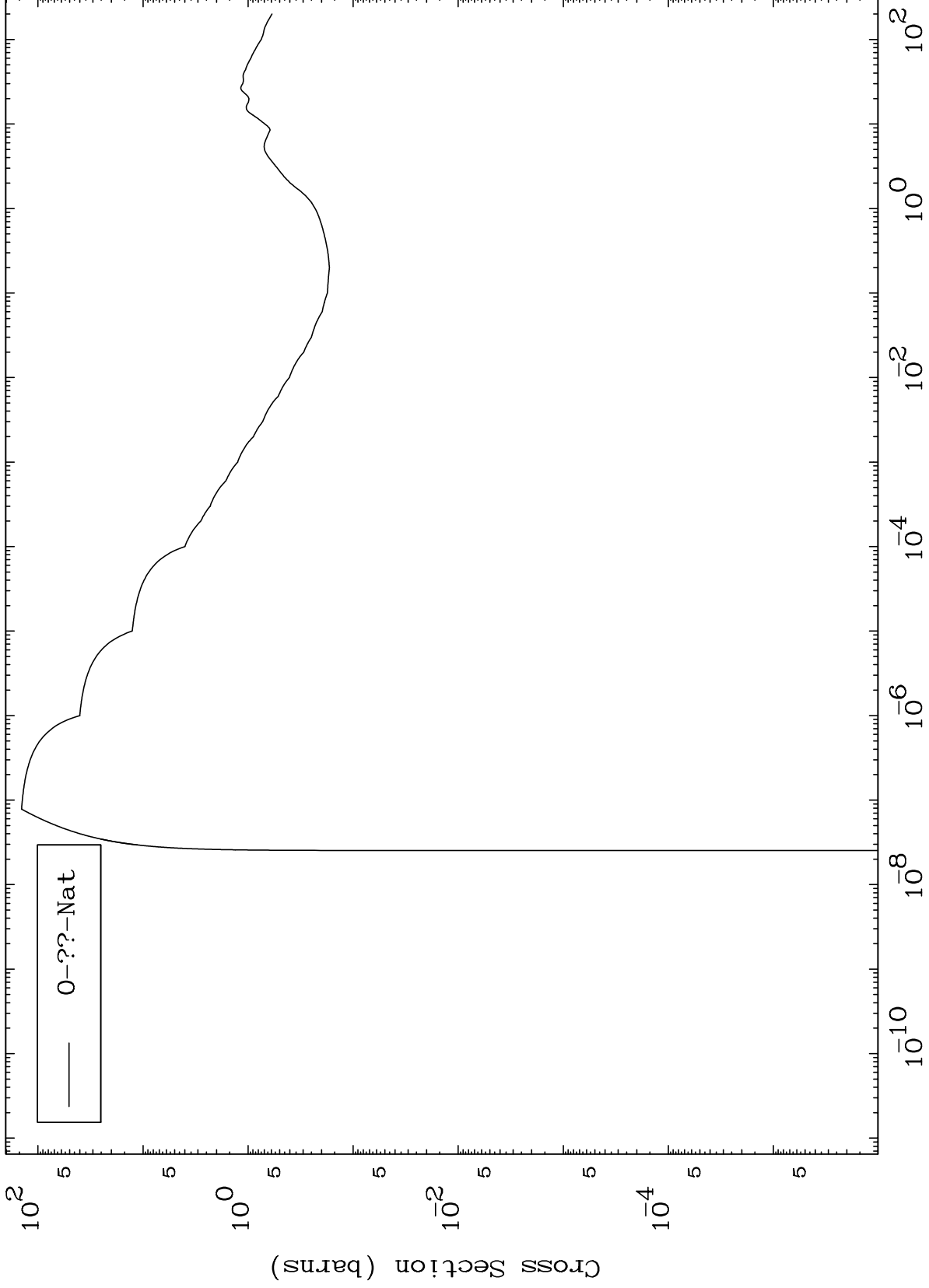
<sup>83</sup>Bi-194m

MAT 8281

Fission

<sup>83</sup>Bi-194m

Radionuclide Production Cross Section



23

Incident Energy (MeV)

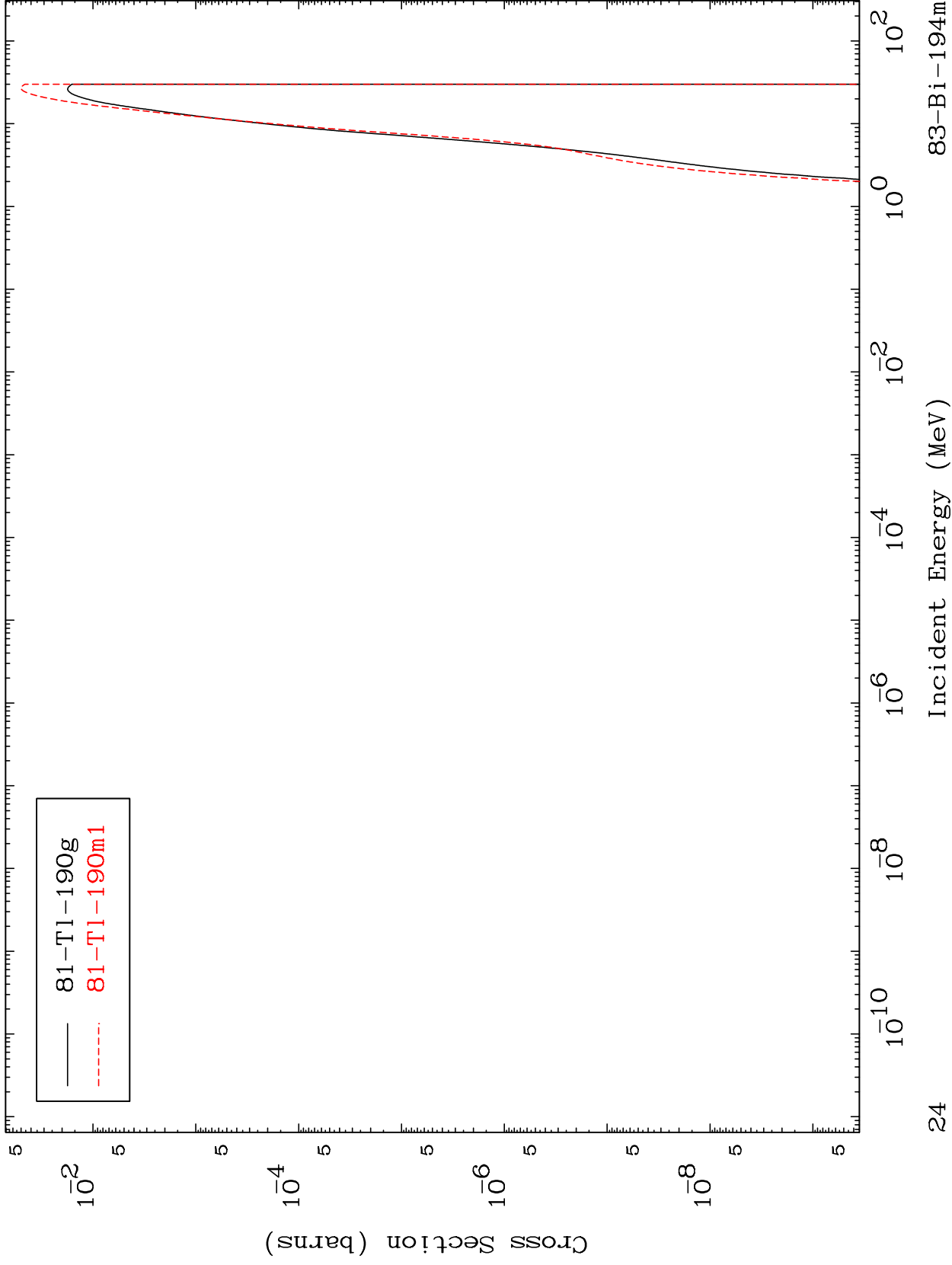
<sup>83</sup>Bi-194m

MAT 8281

$(n, n') \alpha$

83-Bi-194m

Radionuclide Production Cross Section

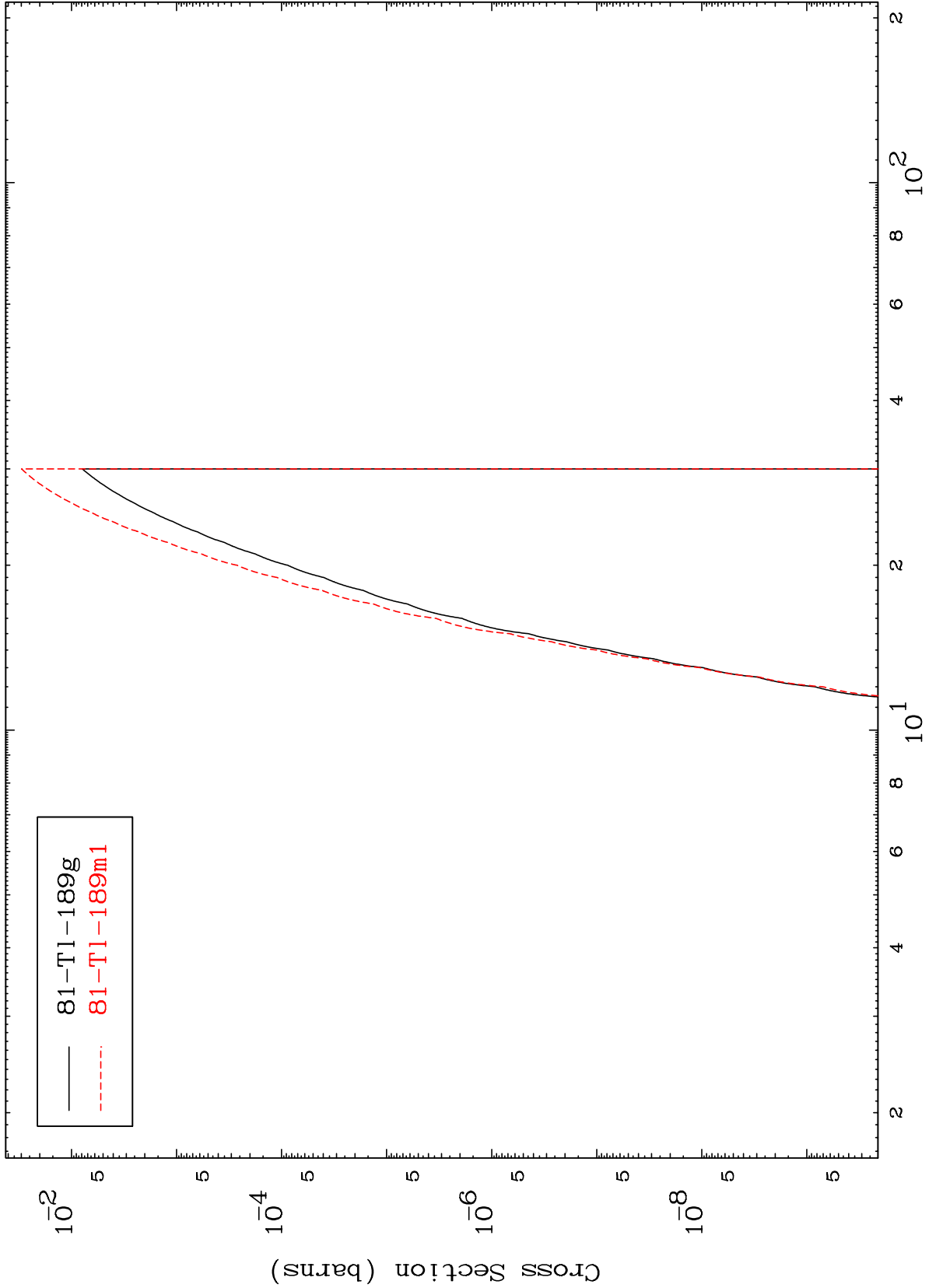


MAT 8281

(n,2n)  $\alpha$

83-Bi-194m

Radionuclide Production Cross Section



25

Incident Energy (MeV)

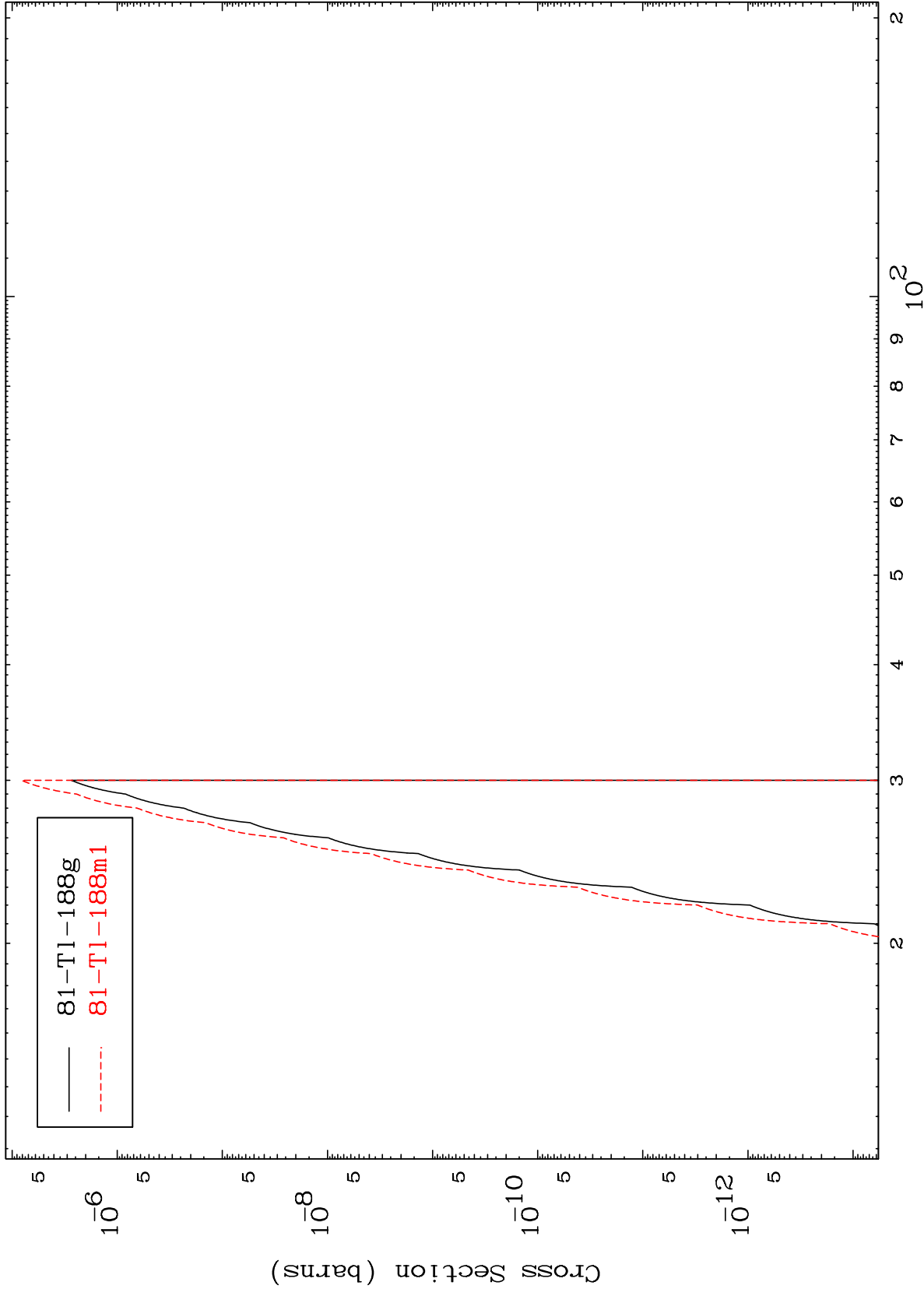
83-Bi-194m

MAT 8281

(n,3n)  $\alpha$

83-Bi-194m

Radionuclide Production Cross Section



26

Incident Energy (MeV)

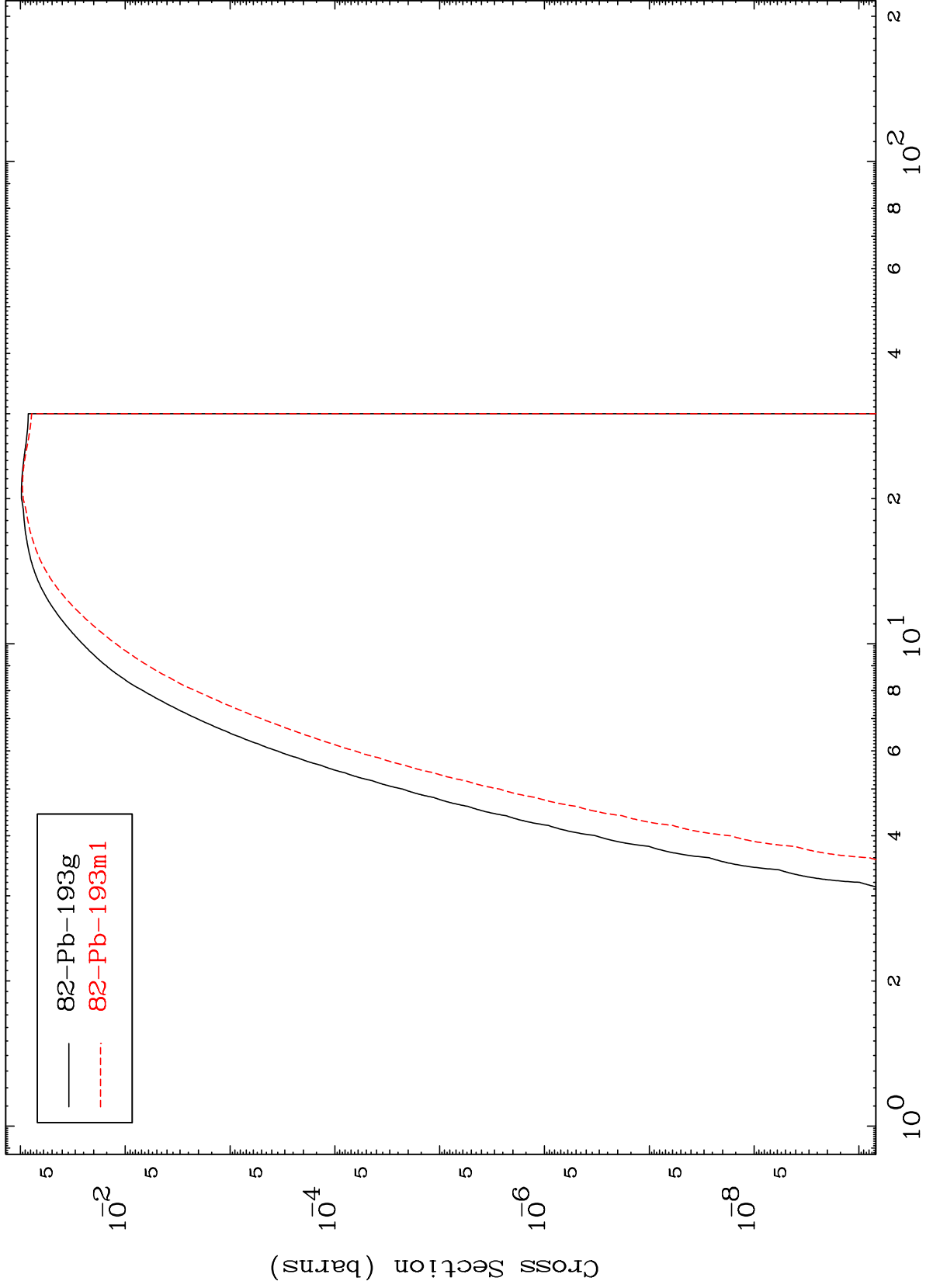
83-Bi-194m

MAT 8281

(n,n') p

83-Bi-194m

Radionuclide Production Cross Section



27

Incident Energy (MeV)

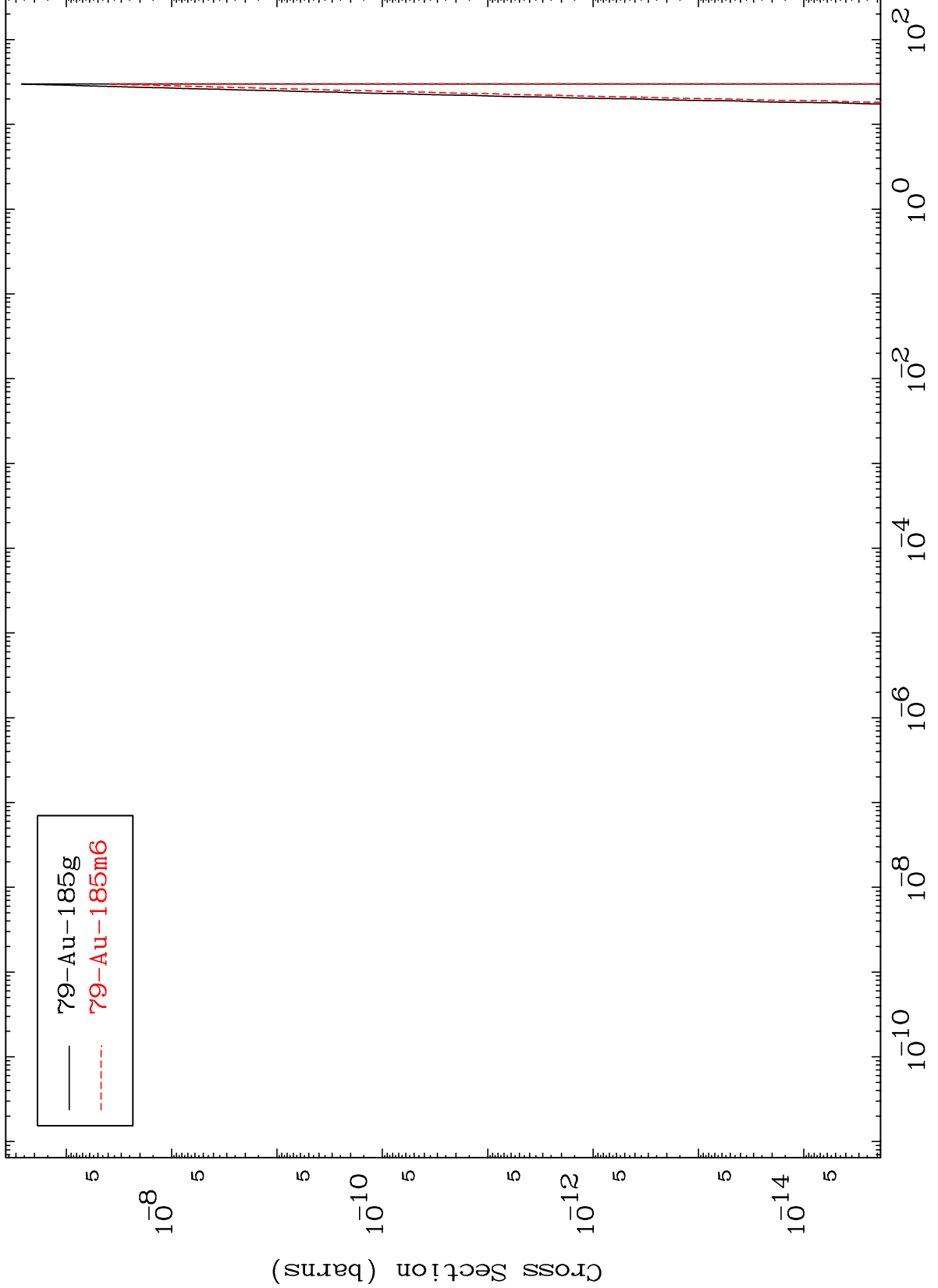
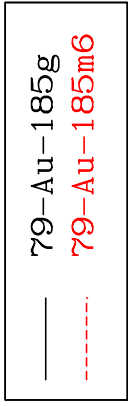
83-Bi-194m

MAT 8281

(n,2n) 2 $\alpha$

83-Bi-194m

Radionuclide Production Cross Section

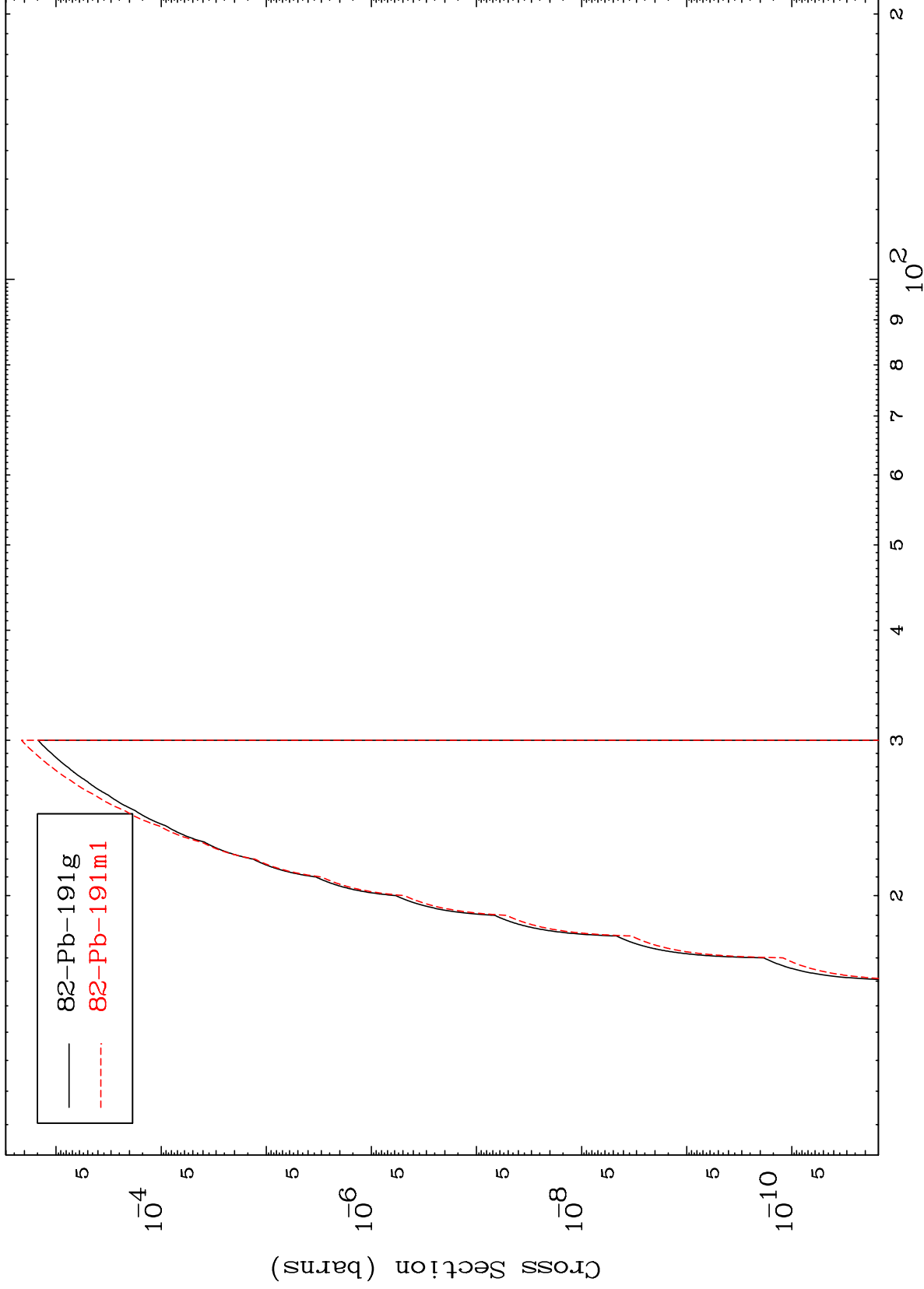


MAT 8281

(n,n') t

83-Bi-194m

Radionuclide Production Cross Section



29

Incident Energy (MeV)

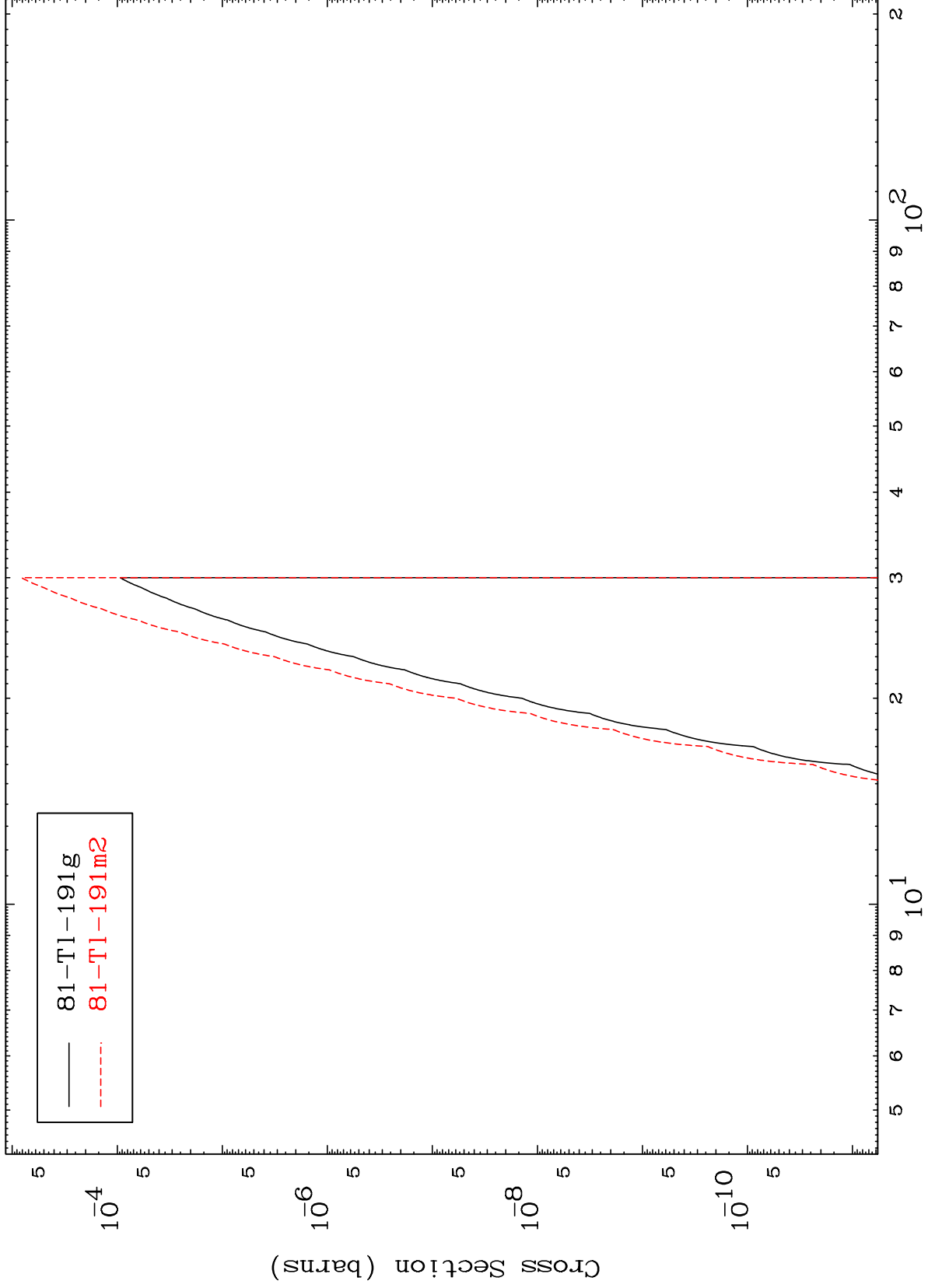
83-Bi-194m

MAT 8281

(n,n') He-3

83-Bi-194m

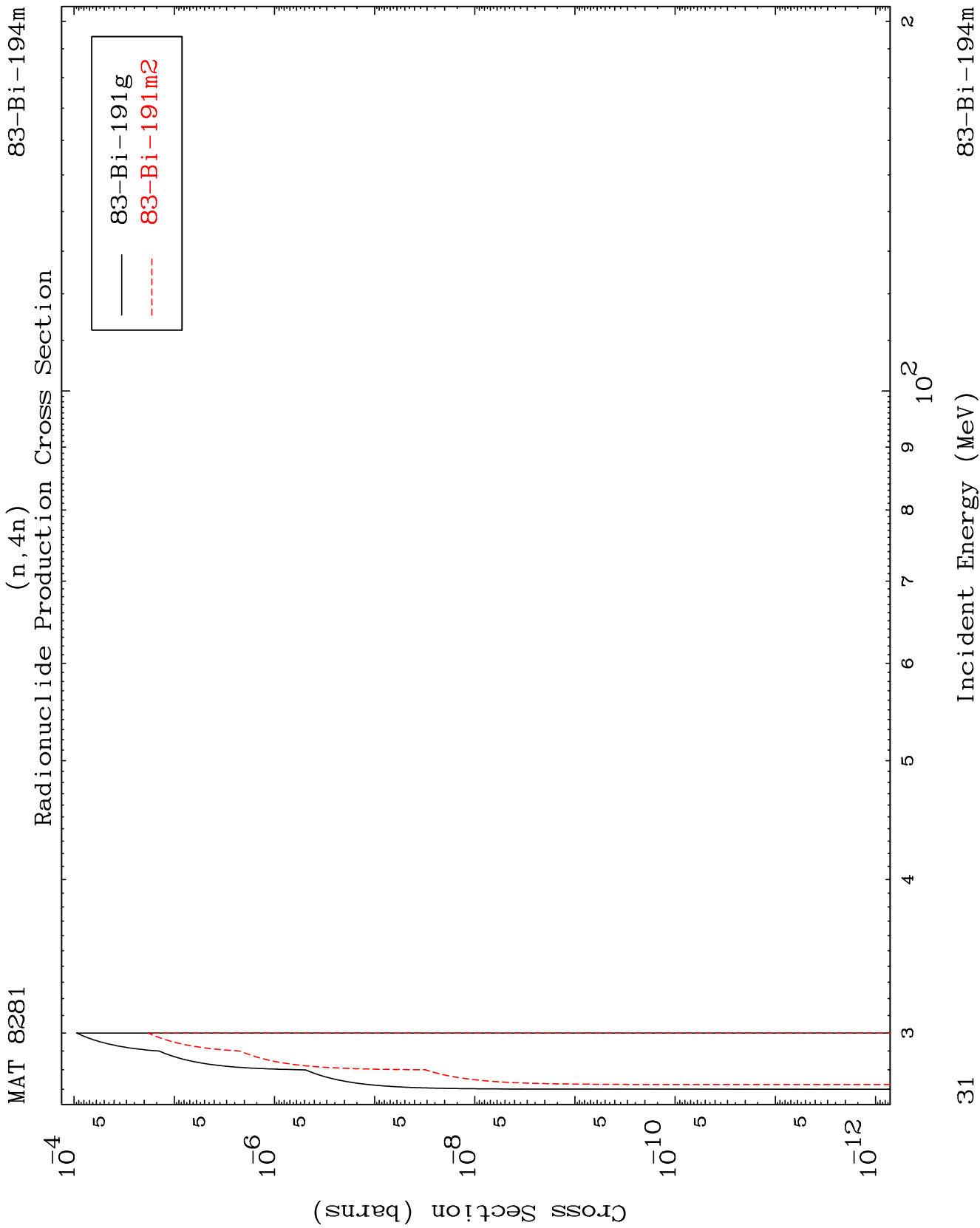
Radionuclide Production Cross Section



30

Incident Energy (MeV)

83-Bi-194m

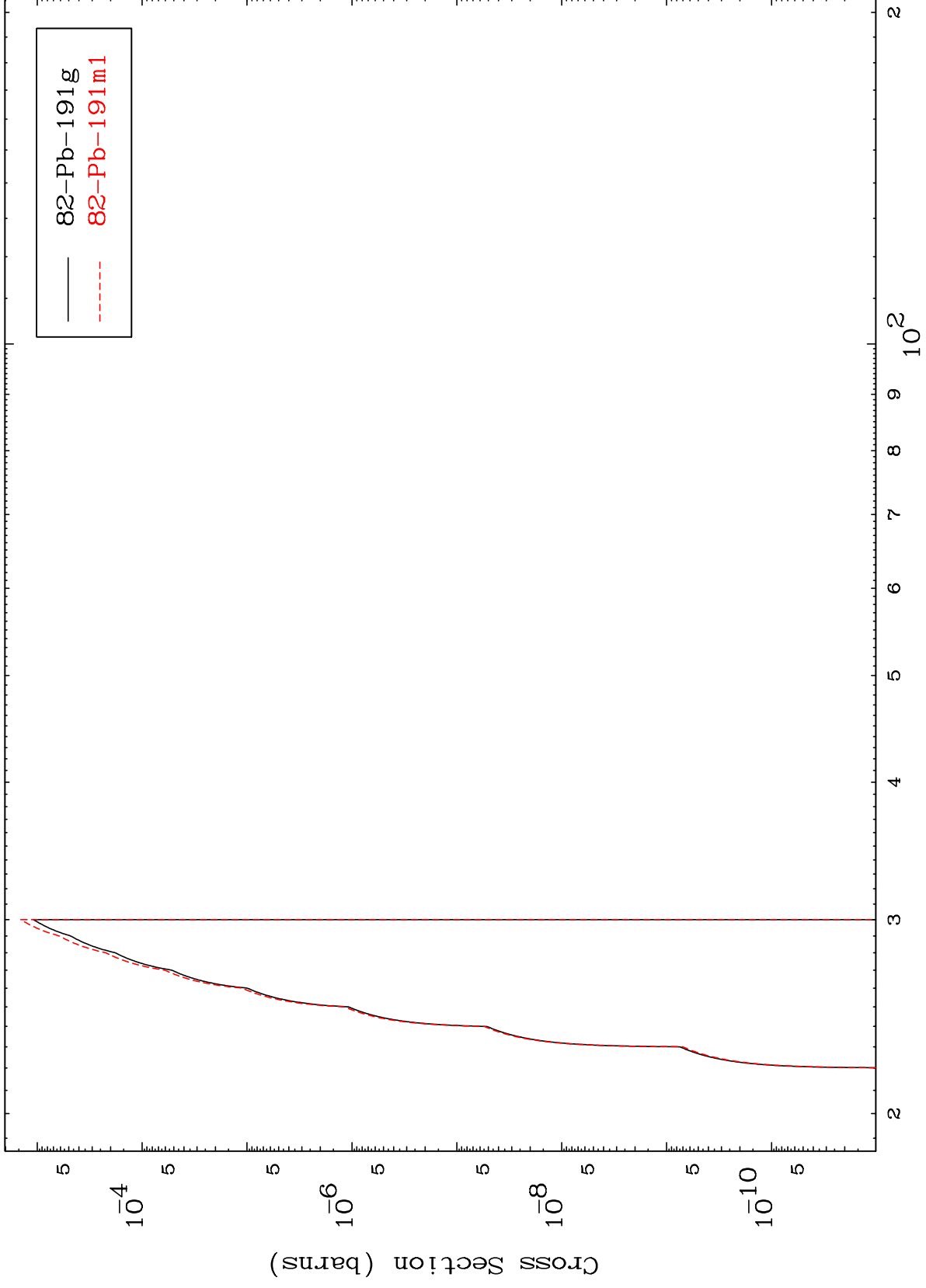


MAT 8281

(n,3n) p

83-Bi-194m

Radionuclide Production Cross Section



32

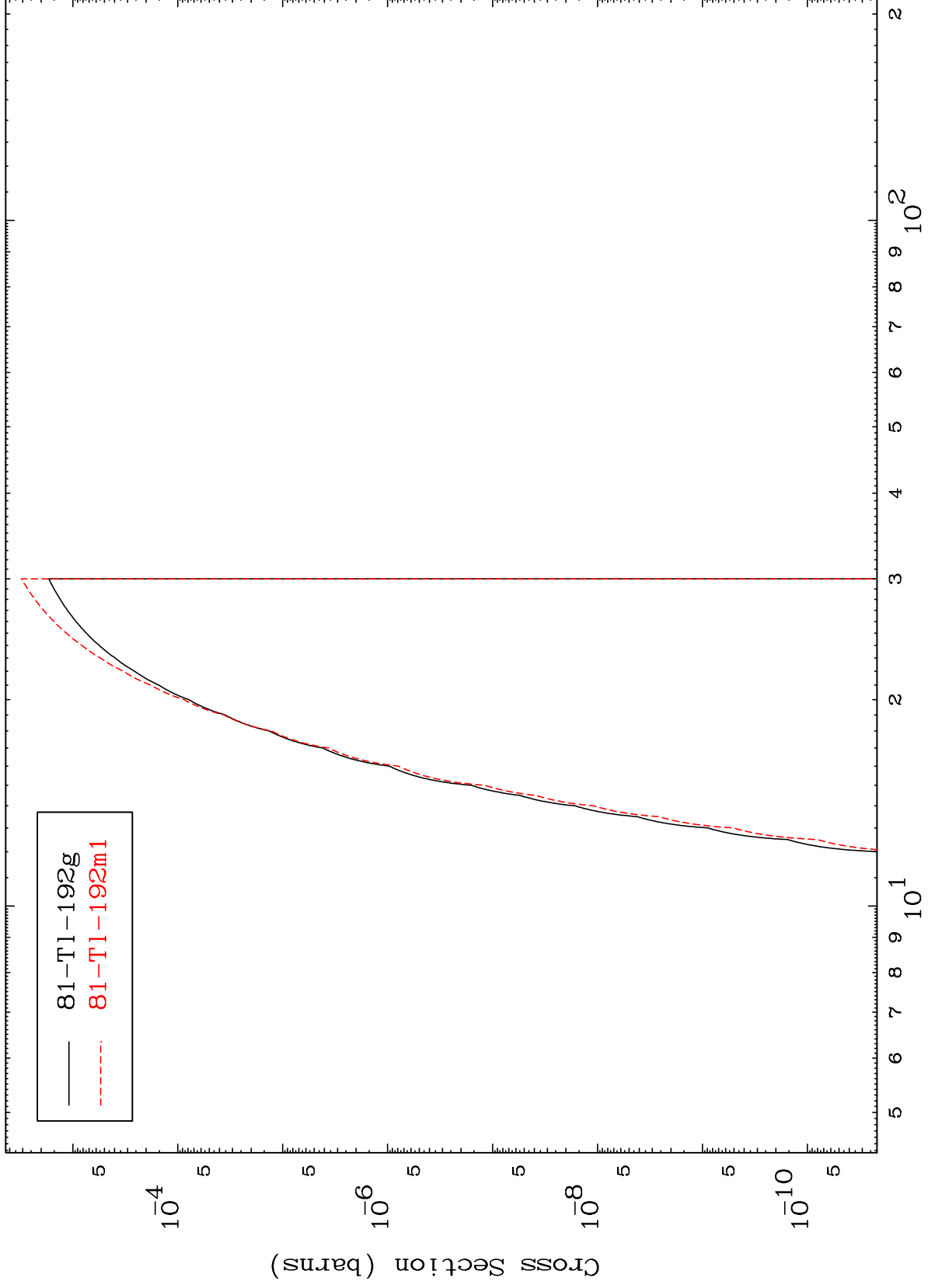
Incident Energy (MeV)

83-Bi-194m

MAT 8281

<sup>83</sup>Bi-194m

(n,2n) p  
Radionuclide Production Cross Section



33

Incident Energy (MeV)

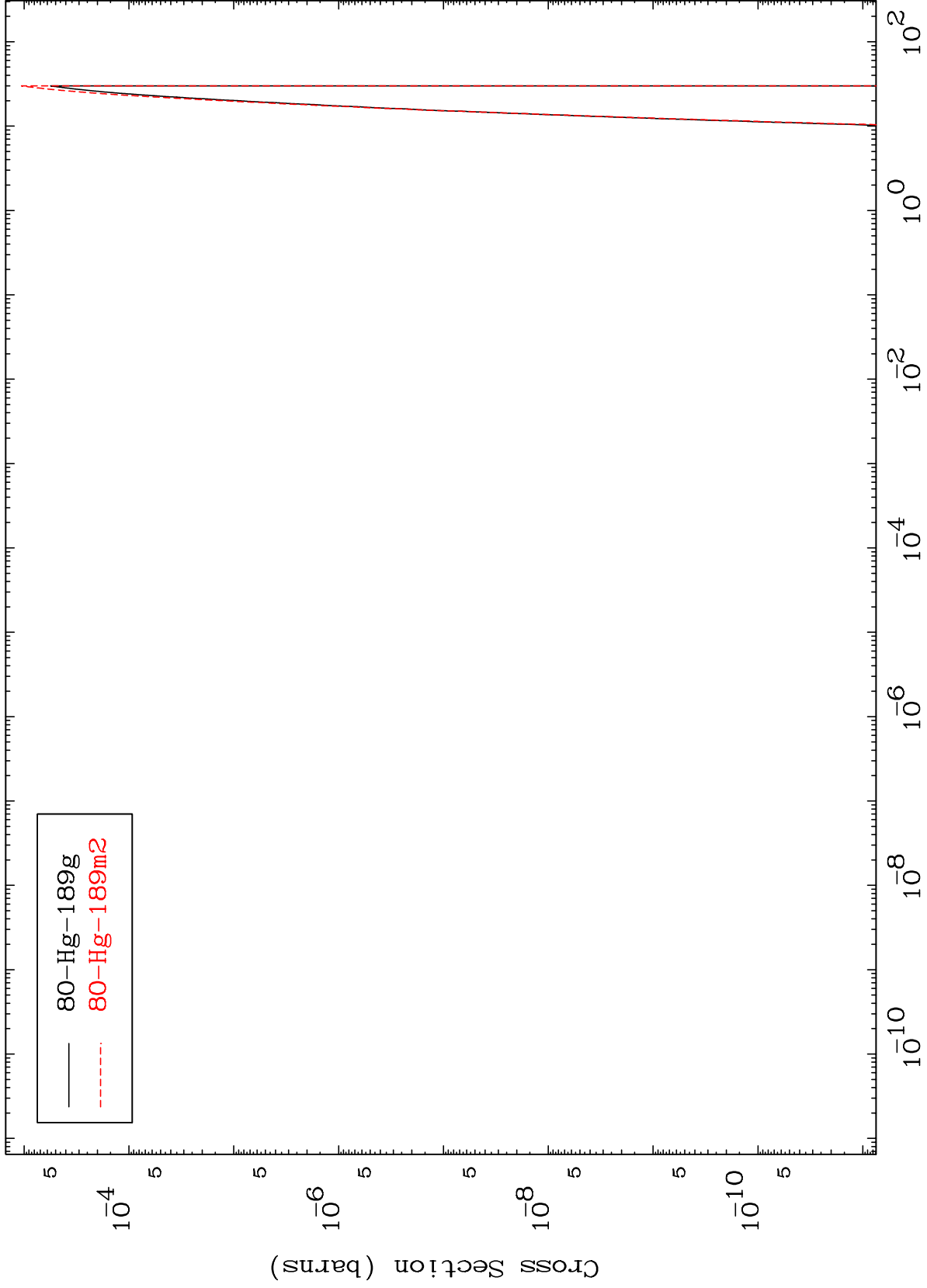
<sup>83</sup>Bi-194m

MAT 8281

(n,n') p  $\alpha$

83-Bi-194m

Radionuclide Production Cross Section



34

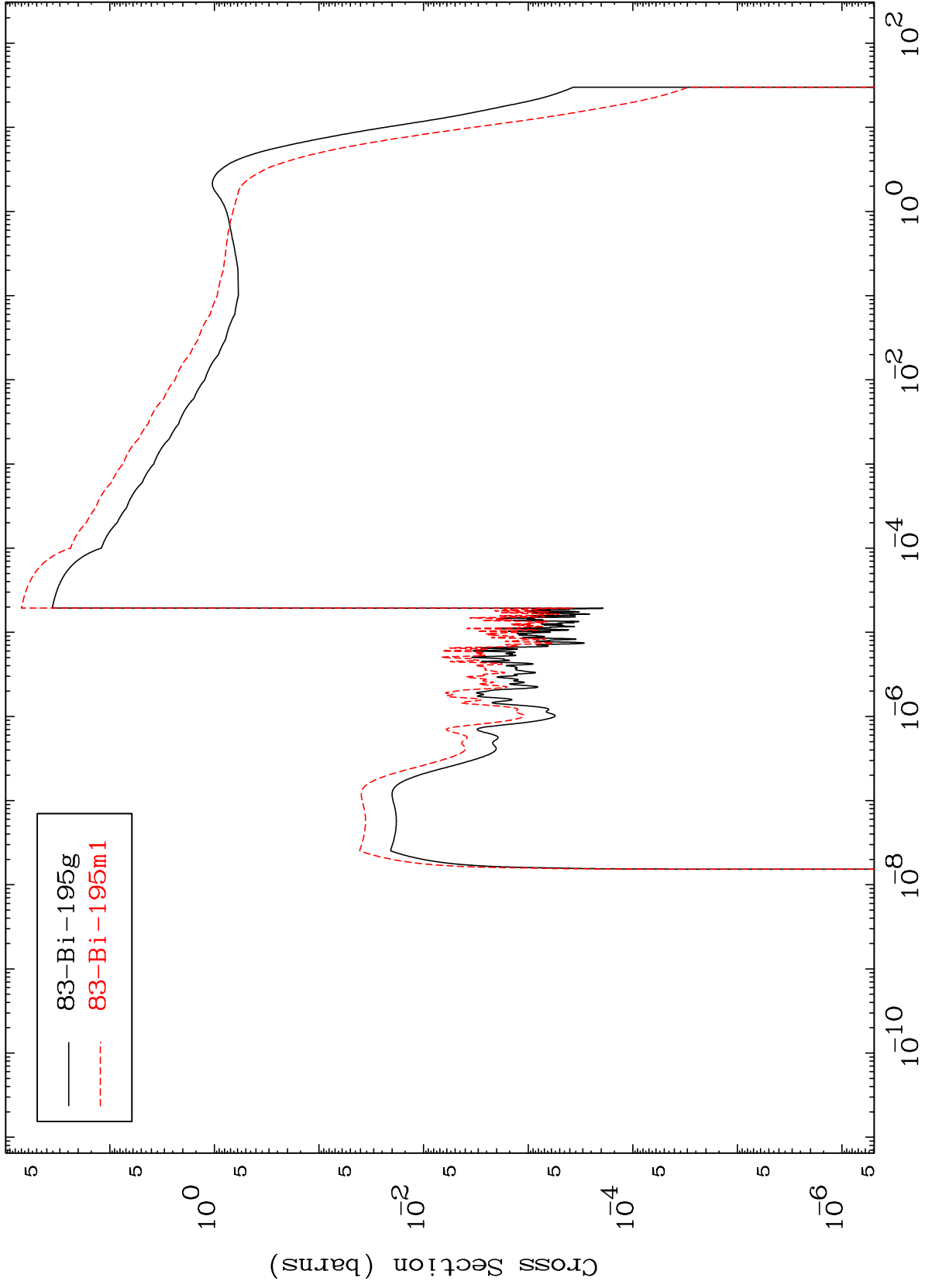
Incident Energy (MeV)

83-Bi-194m

MAT 8281

$^{83}\text{Bi}-194\text{m}$

Radionuclide Production Cross Section



35

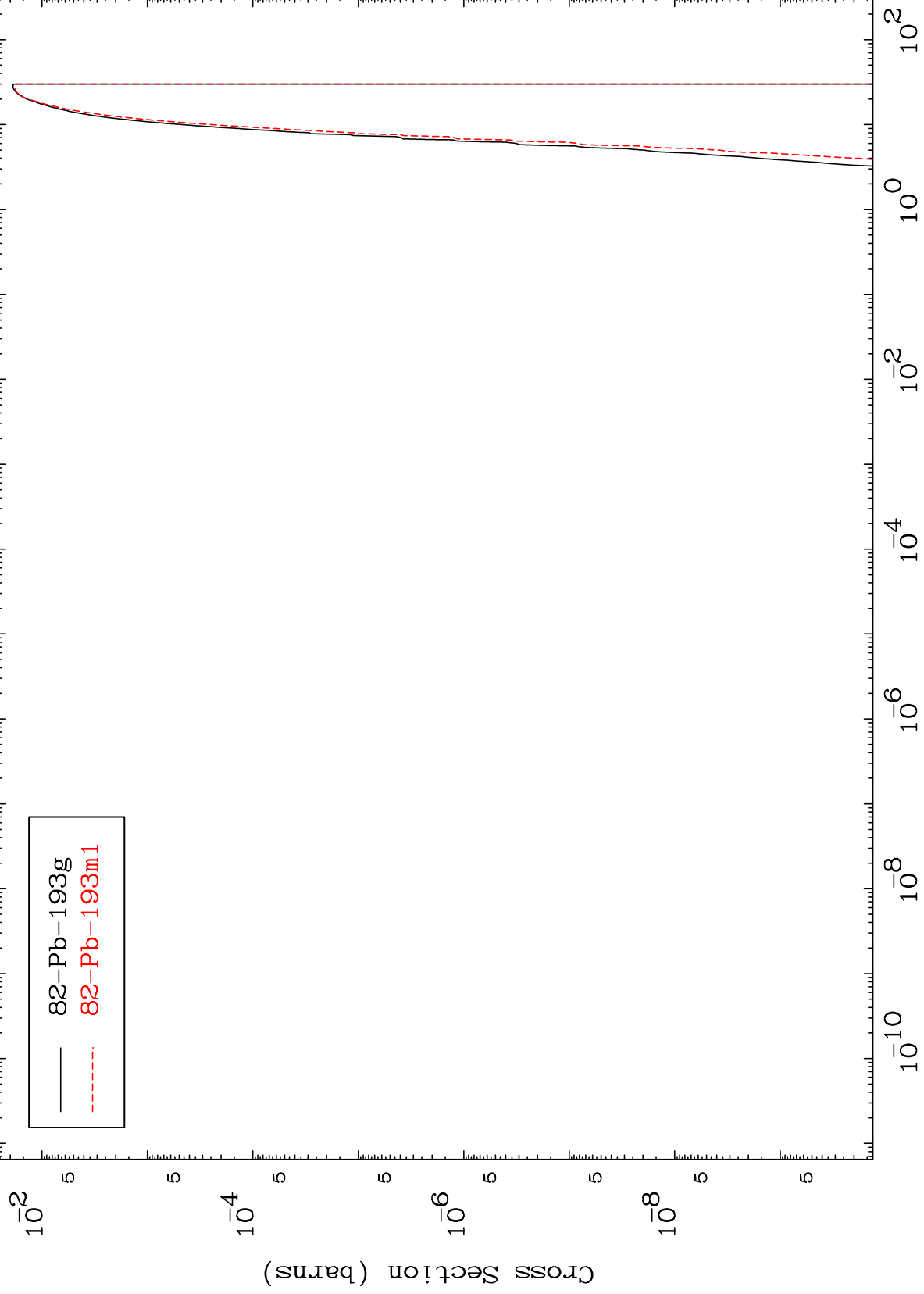
$^{83}\text{Bi}-194\text{m}$

MAT 8281

(n,d)

83-Bi-194m

Radionuclide Production Cross Section



36

Incident Energy (MeV)

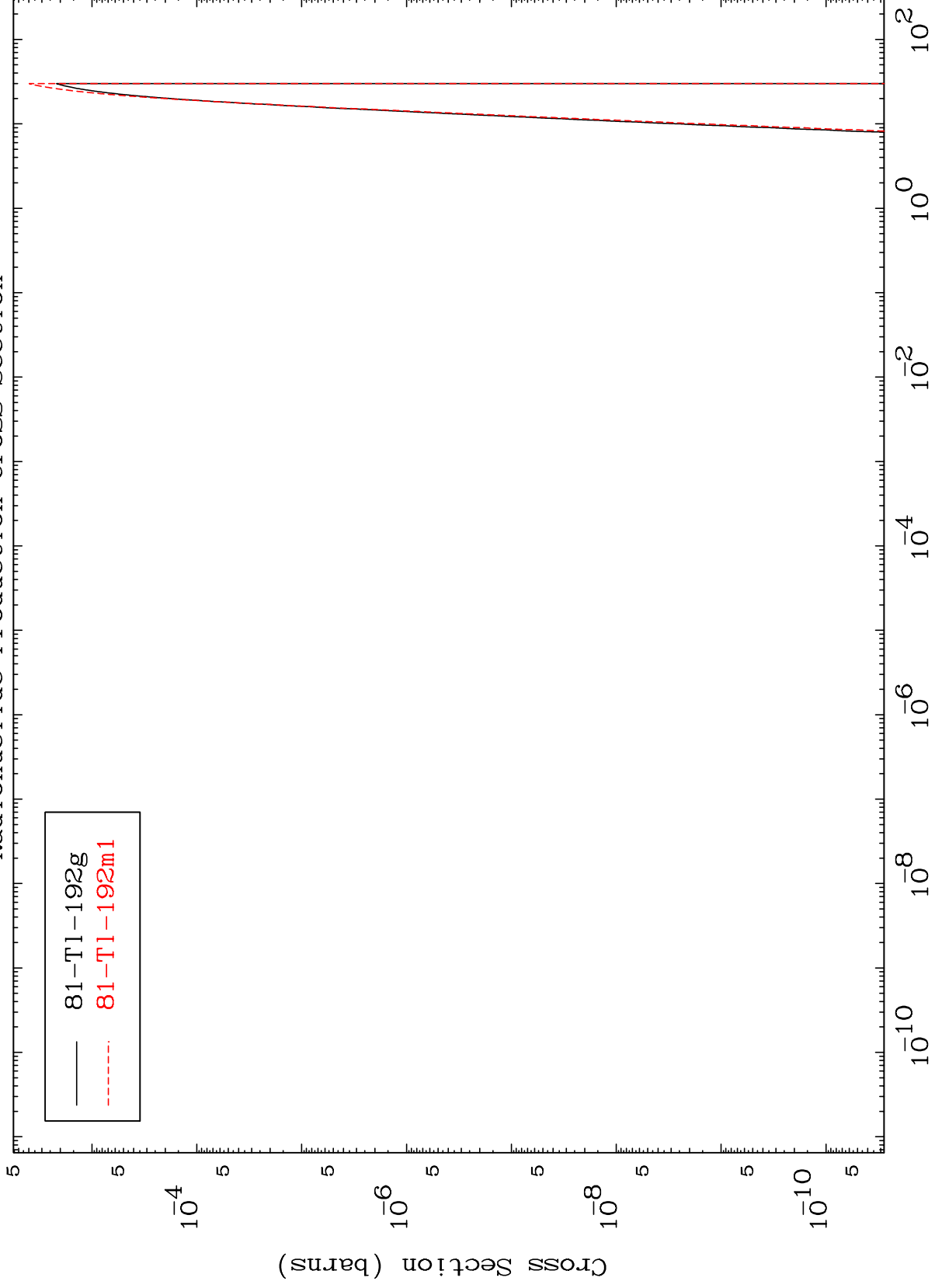
83-Bi-194m

MAT 8281

(n,He-3)

83-Bi-194m

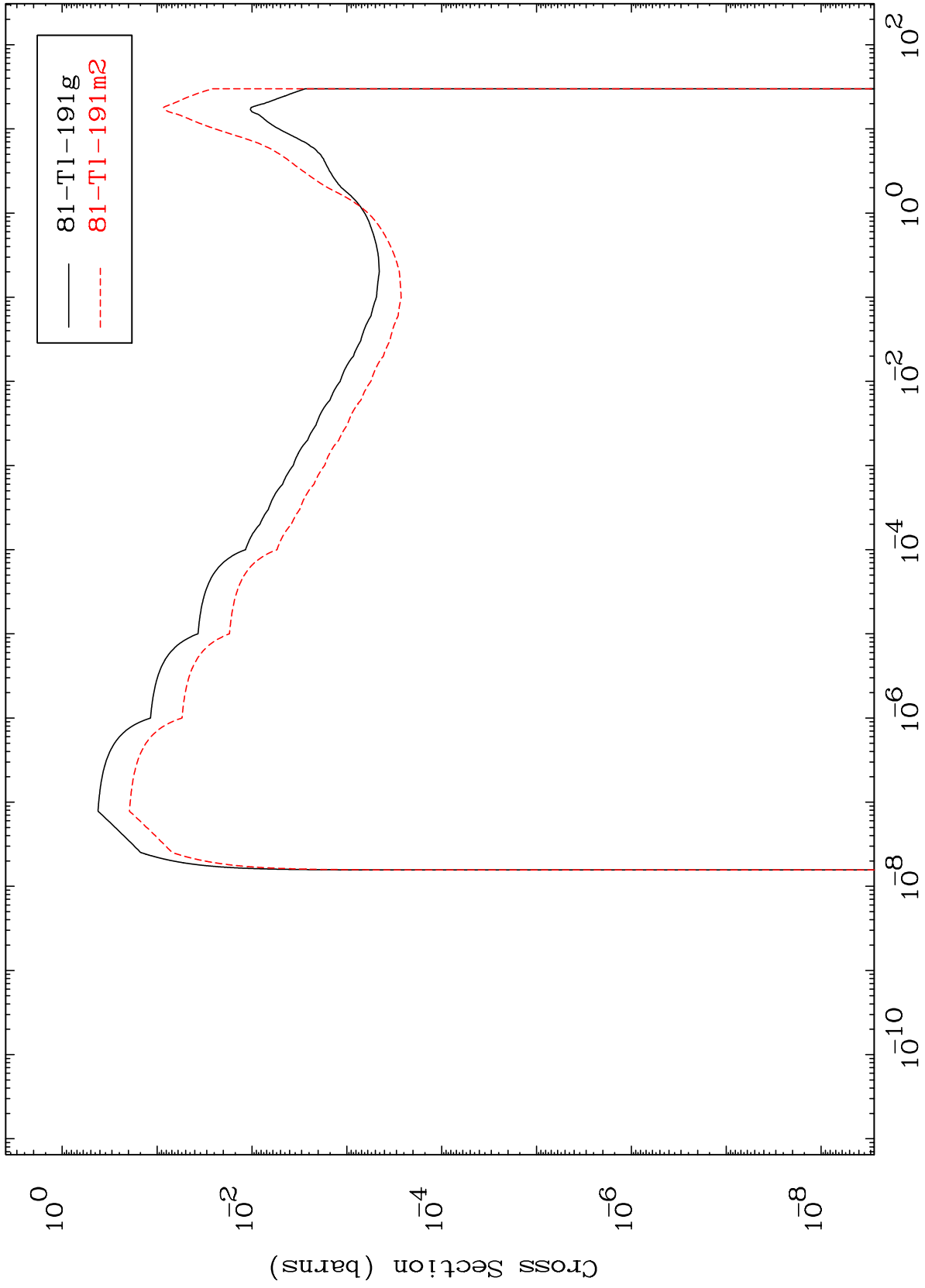
Radionuclide Production Cross Section



MAT 8281

83-Bi-194m

(n,  $\alpha$ )  
Radionuclide Production Cross Section



38

83-Bi-194m

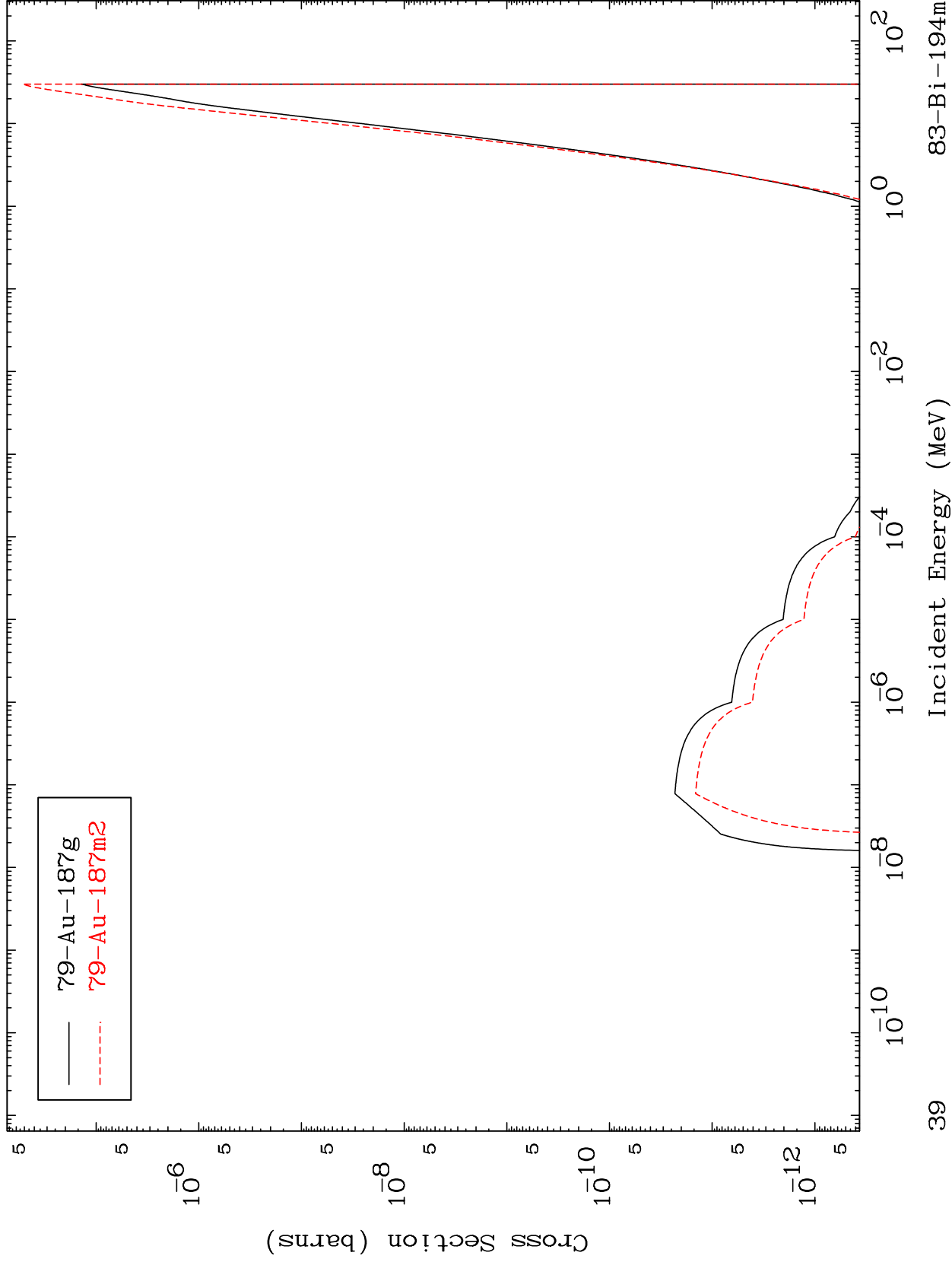
Incident Energy (MeV)

MAT 8281

(n,2α)

<sup>83</sup>Bi-194m

Radionuclide Production Cross Section

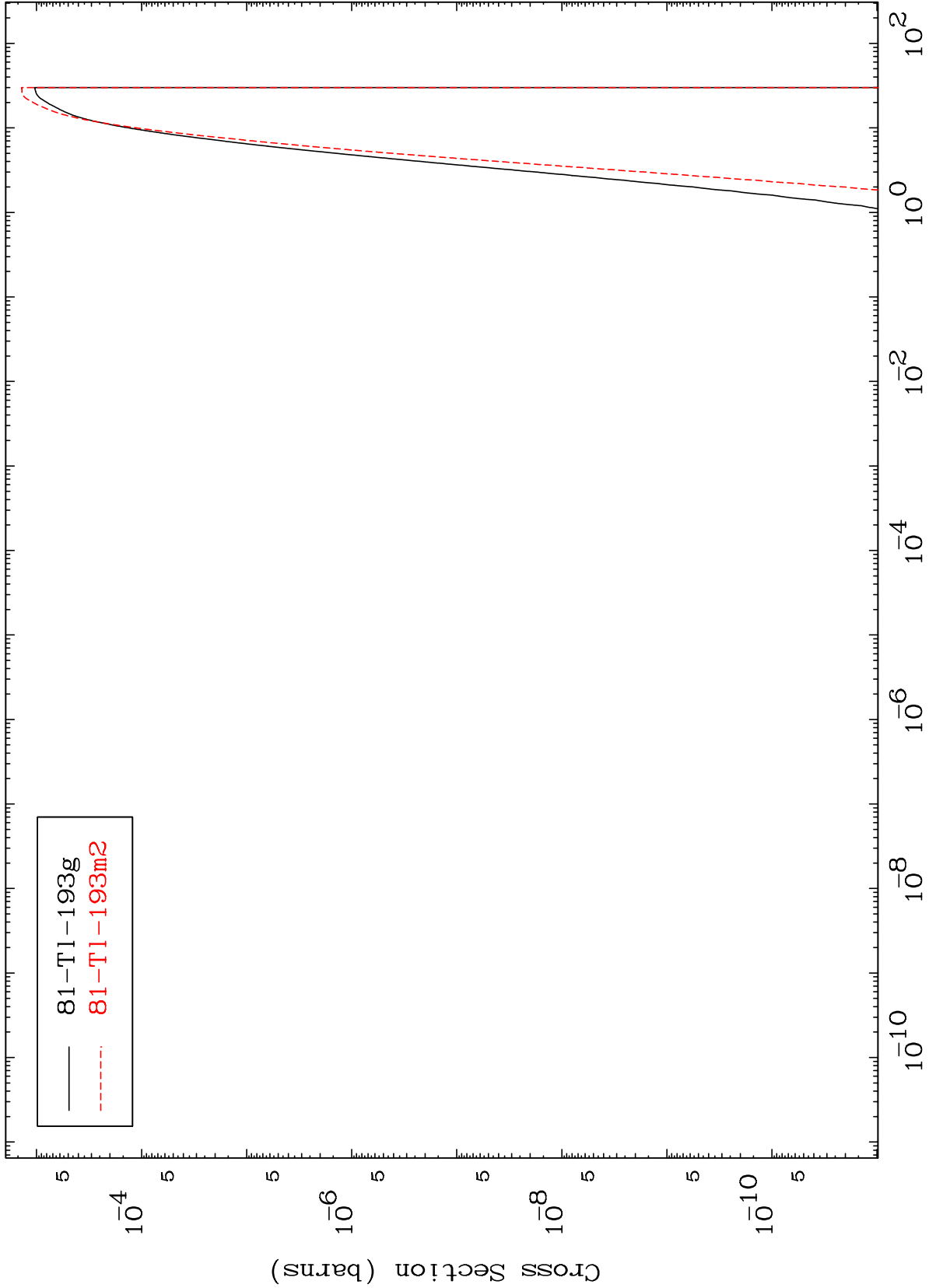


MAT 8281

(n,2p)

83-Bi-194m

Radionuclide Production Cross Section



40

Incident Energy (MeV)

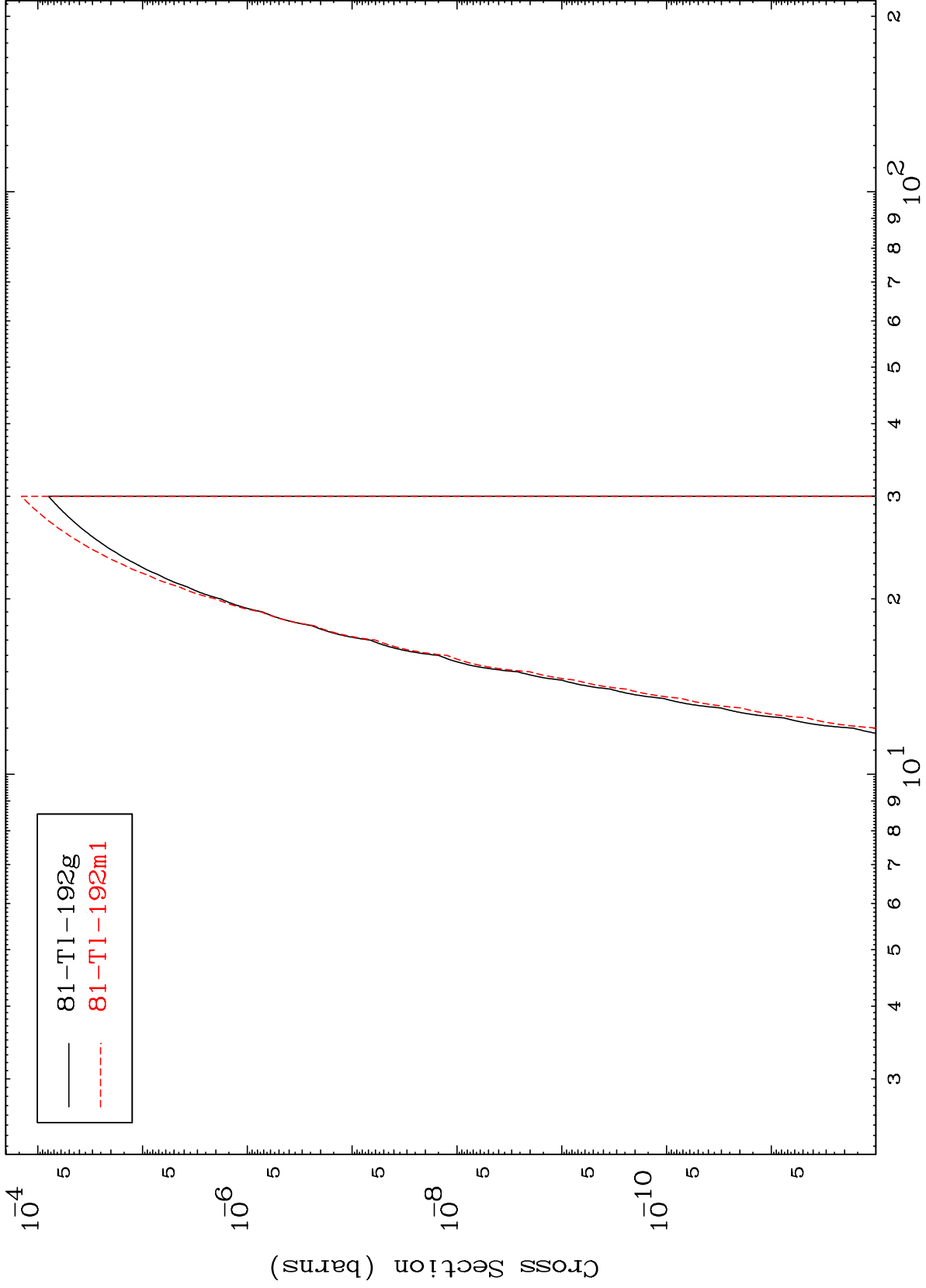
83-Bi-194m

MAT 8281

(n,p) d

83-Bi-194m

Radionuclide Production Cross Section

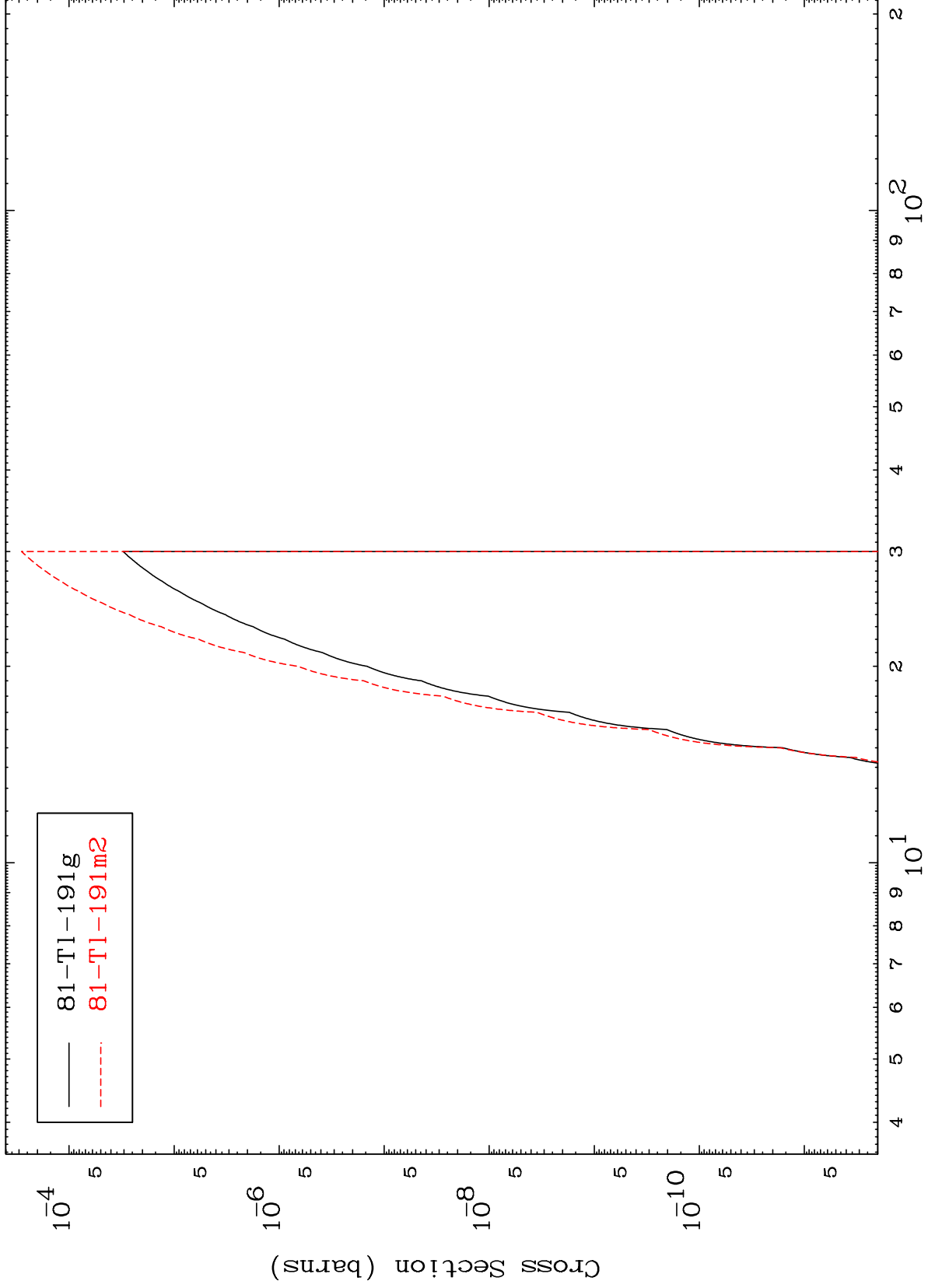


MAT 8281

(n,p) t

83-Bi-194m

Radionuclide Production Cross Section



42

Incident Energy (MeV)

83-Bi-194m

MAT 8281

(n,d)  $\alpha$

83-Bi-194m

Radionuclide Production Cross Section

