

Program EVALPLOT  
(Version 2021-1)

by

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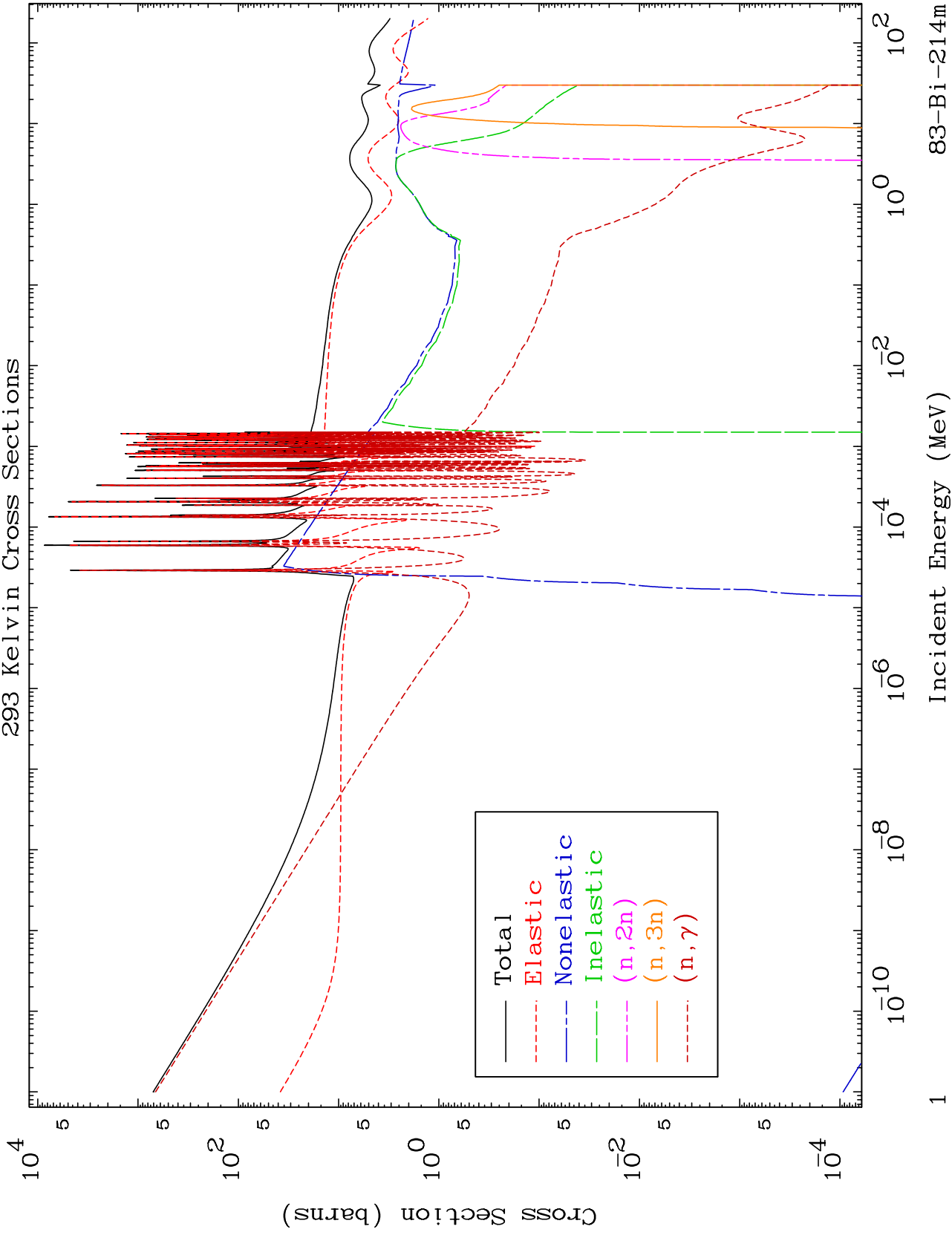
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8341

Neutron Major  
293 Kelvin Cross Sections

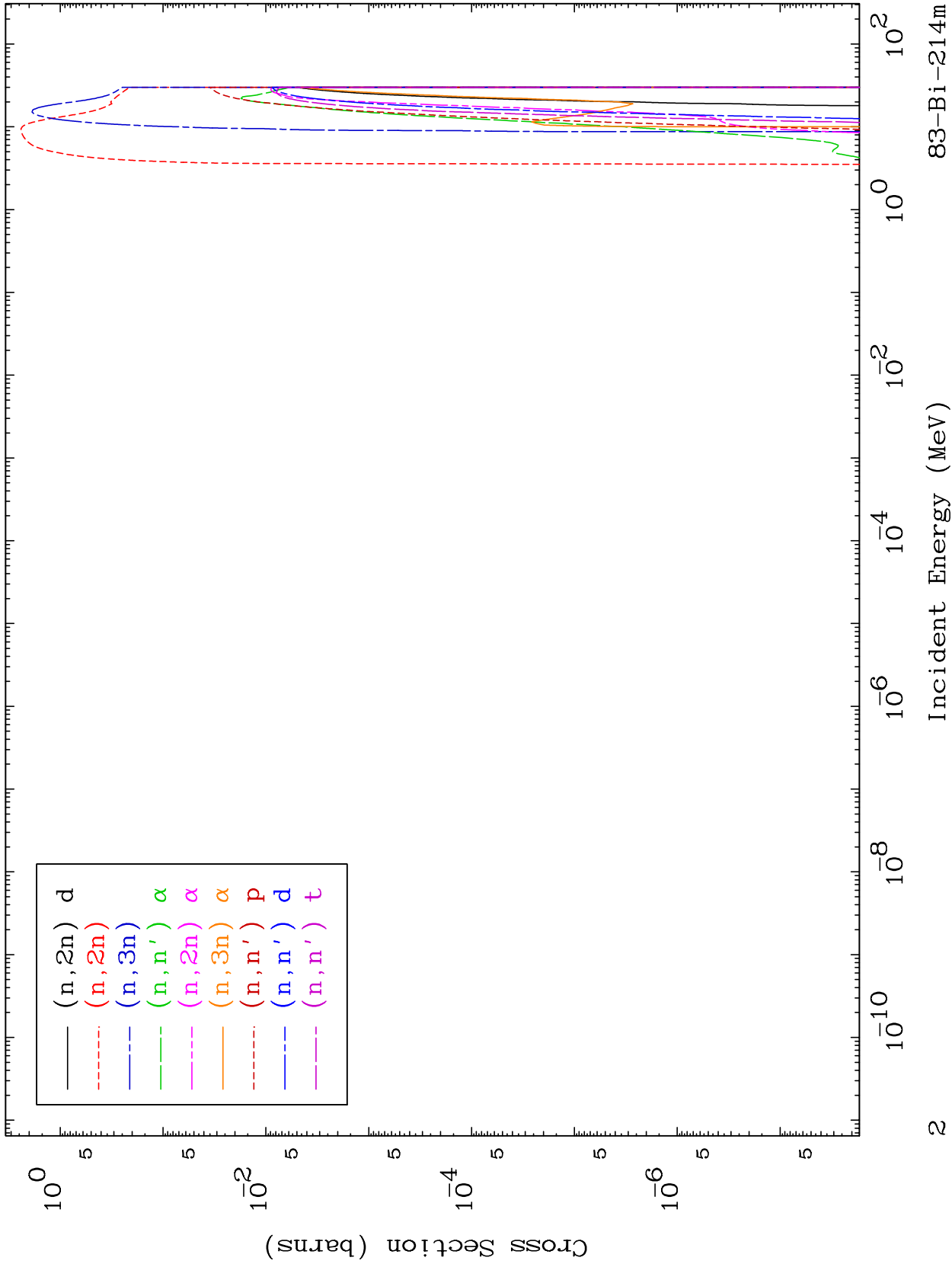
83-Bi-214m



MAT 8341

Neutron Absorption  
293 Kelvin Cross Sections

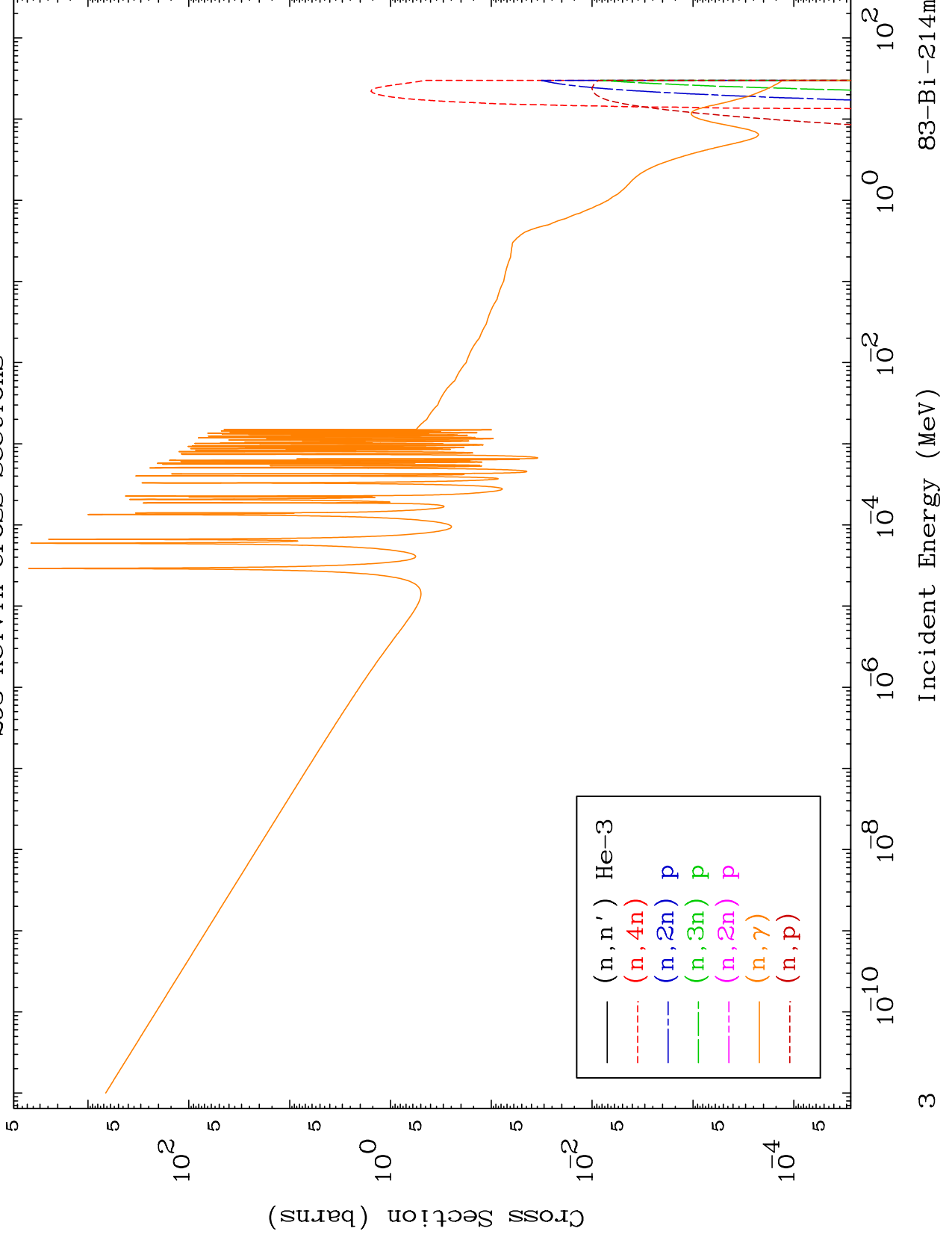
83-Bi-214m



MAT 8341

Neutron Absorption  
293 Kelvin Cross Sections

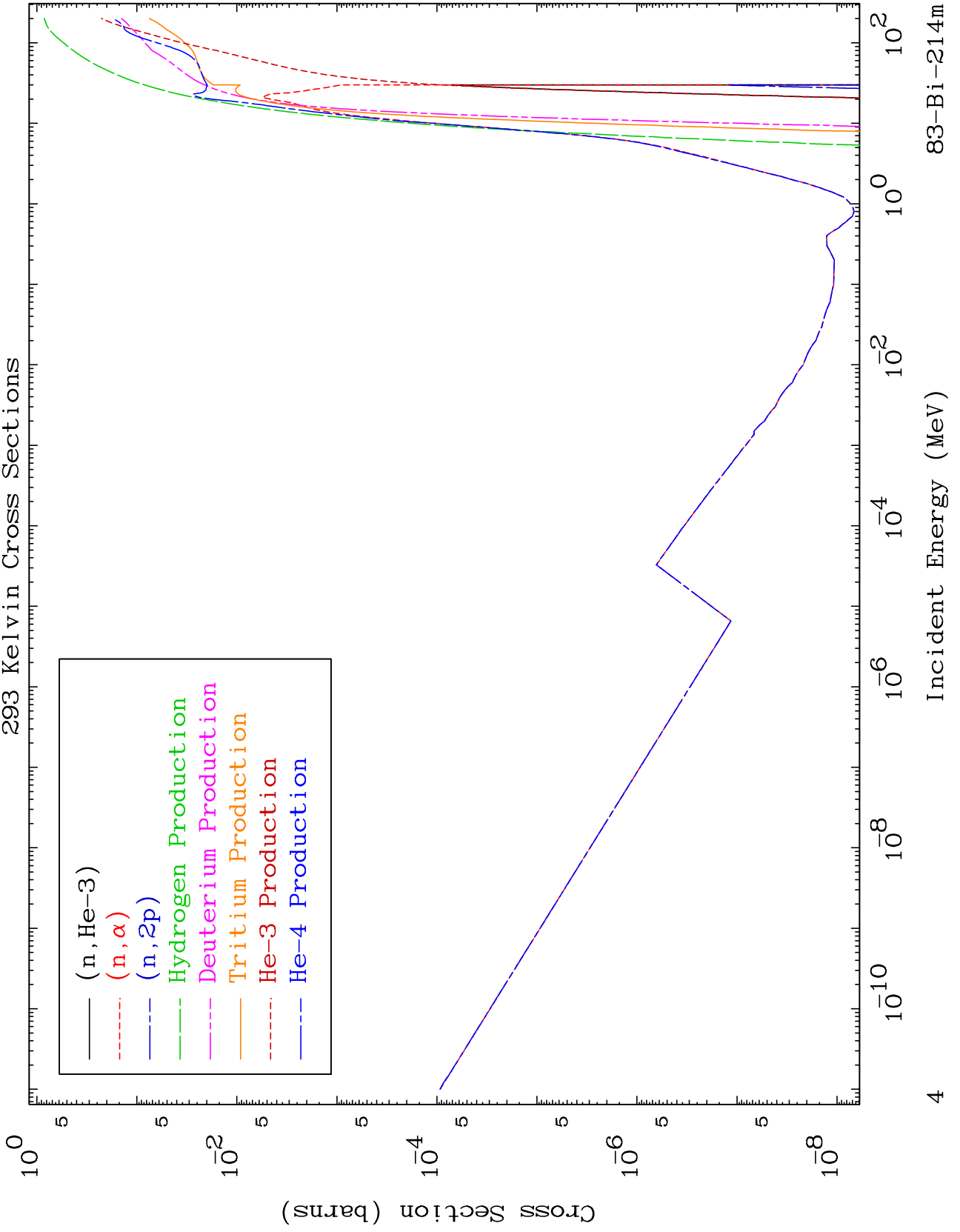
83-Bi-214m



MAT 8341

Neutron Absorption  
293 Kelvin Cross Sections

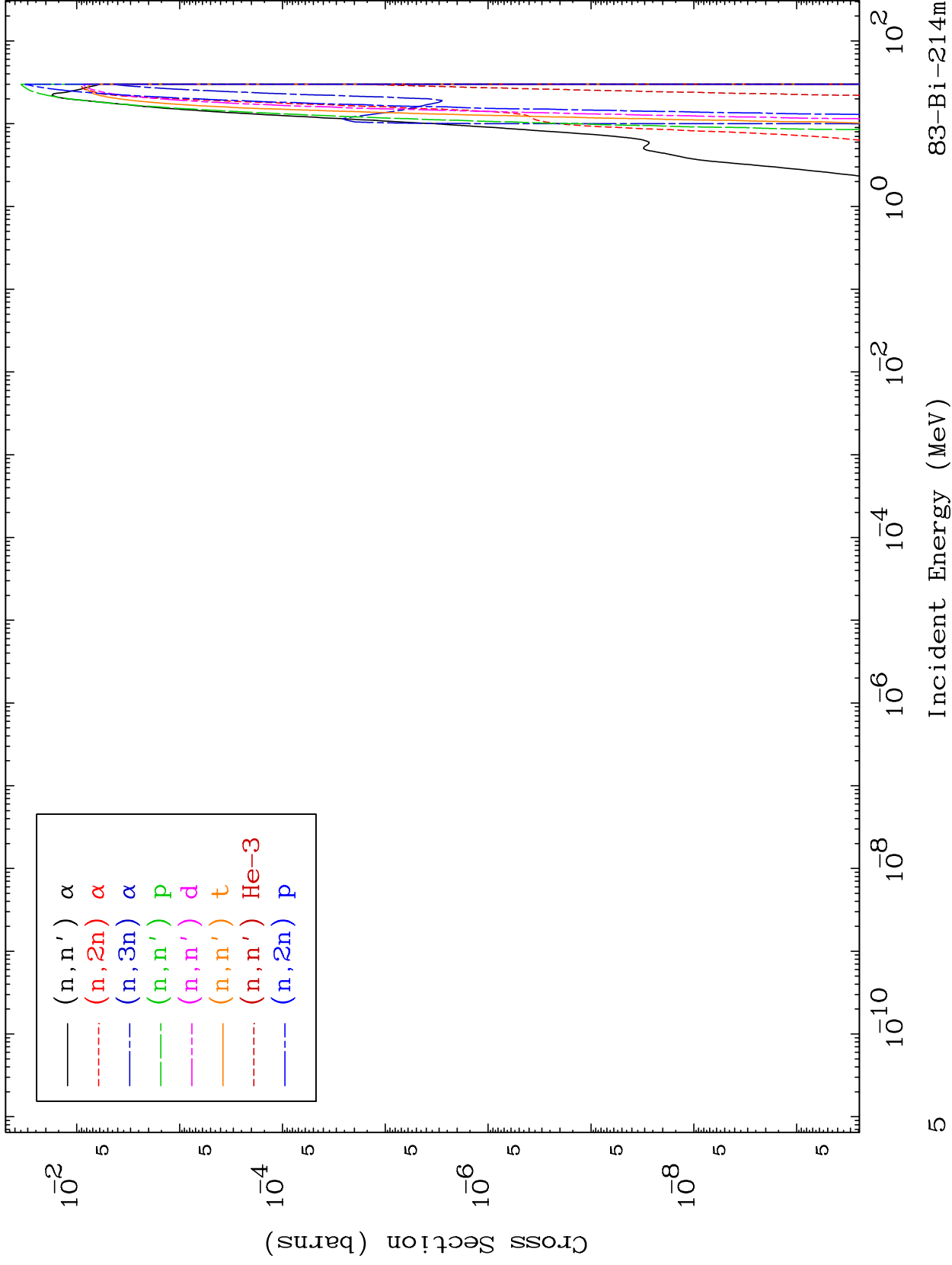
83-Bi-214m



MAT 8341

Charged Particle  
293 Kelvin Cross Sections

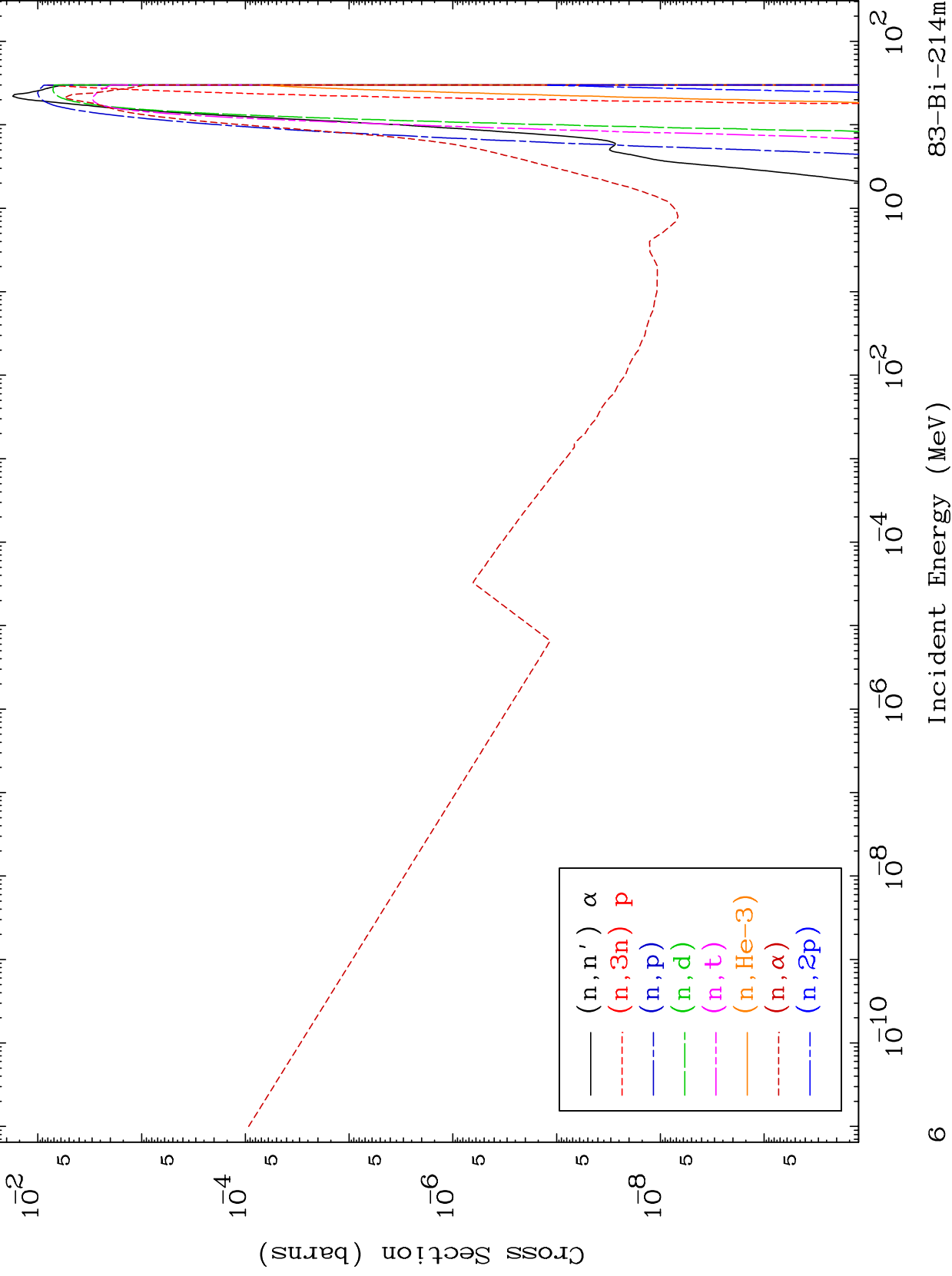
83-Bi-214m



MAT 8341

Charged Particle  
293 Kelvin Cross Sections

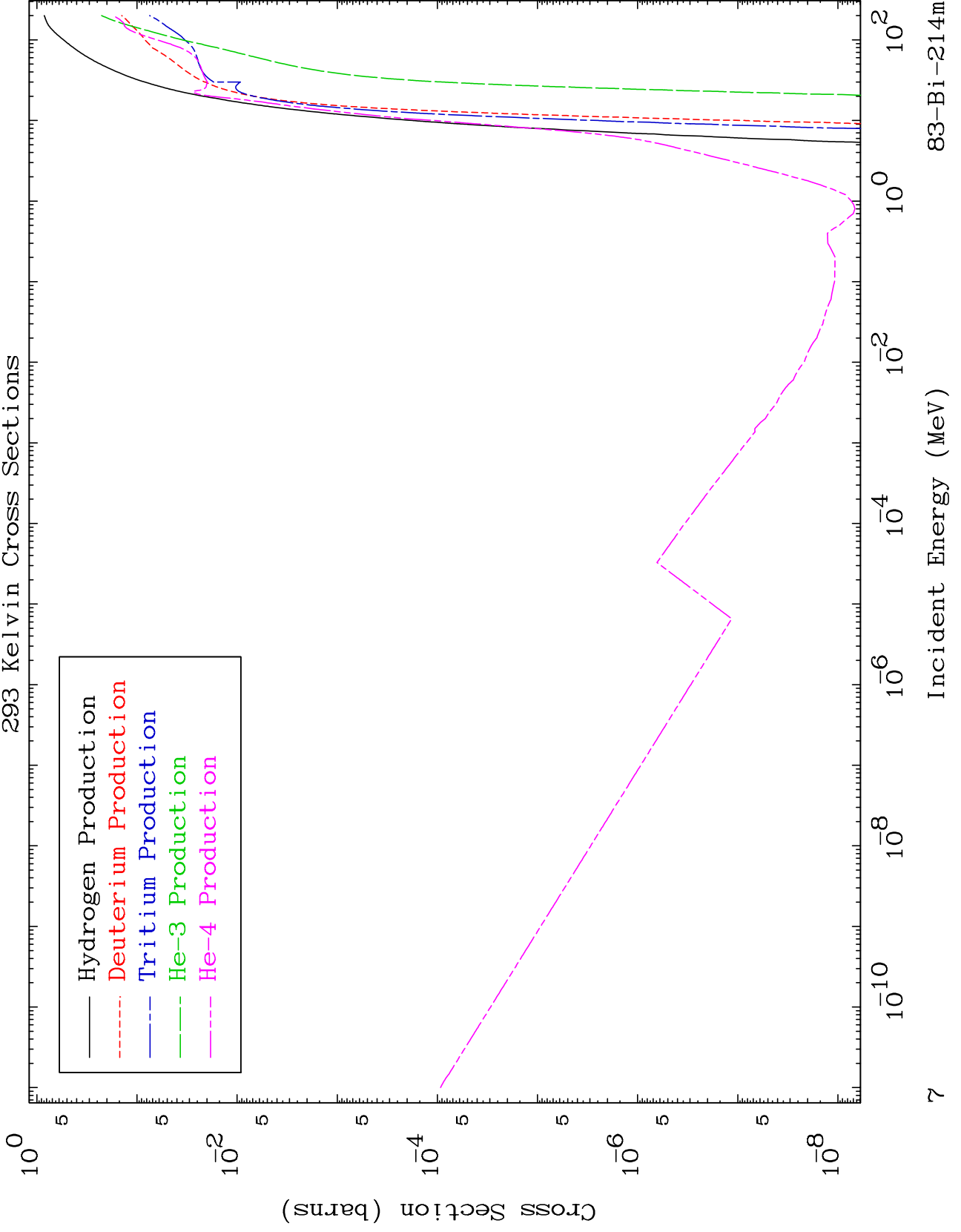
83-Bi-214m



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Particle Production  
293 Kelvin Cross Sections

83-Bi-214m

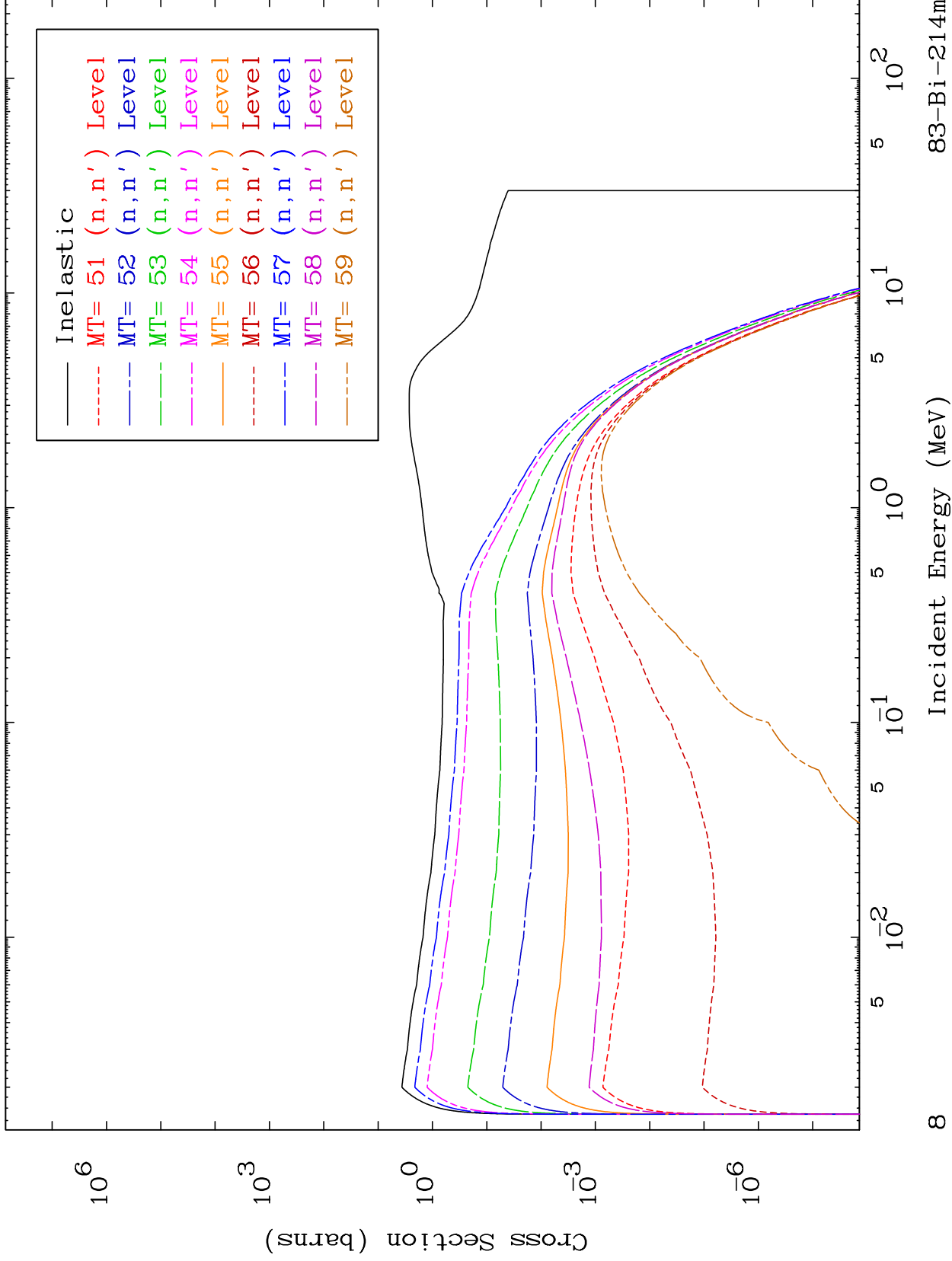


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(n,n') Levels

83-Bi-214m

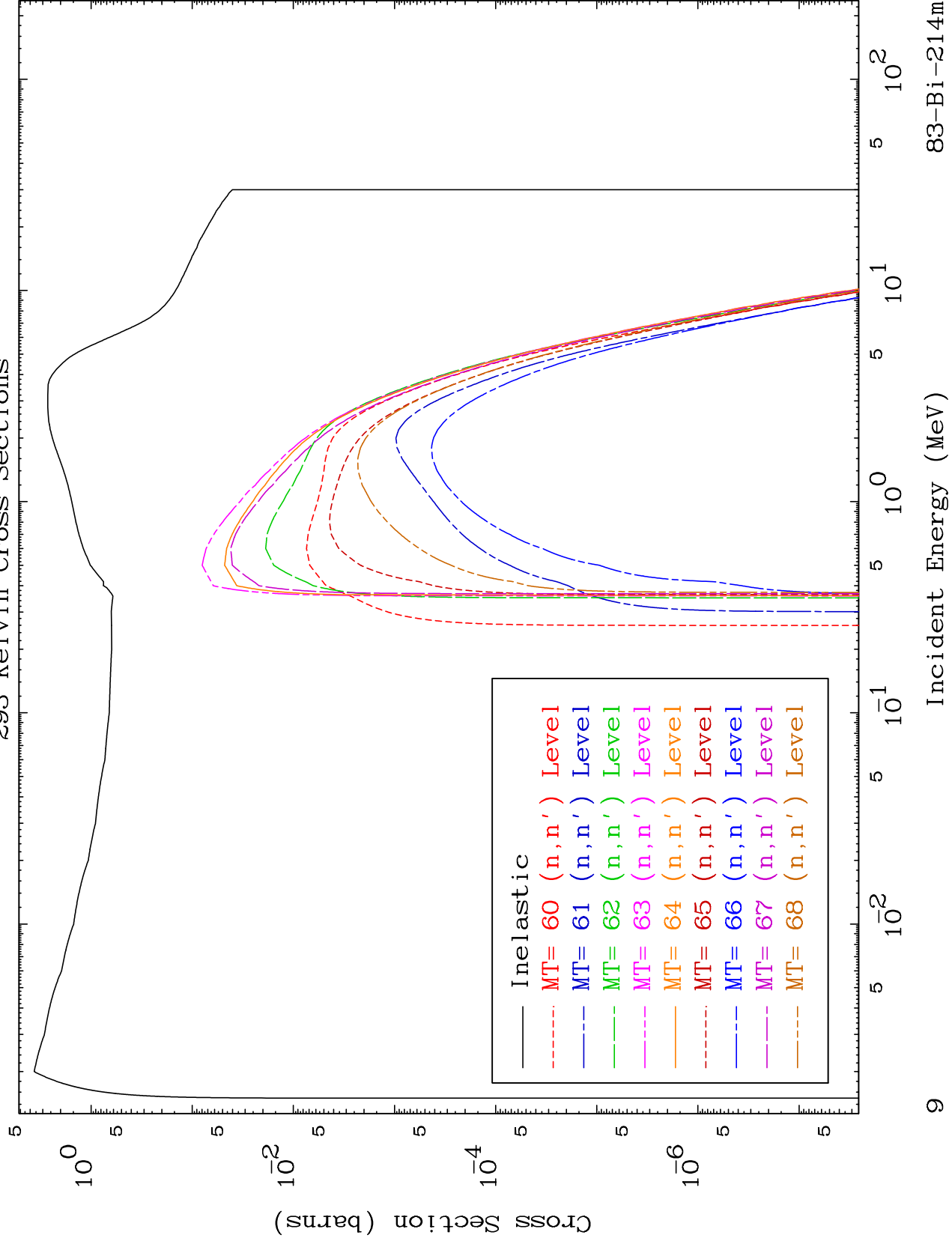
293 Kelvin Cross Sections



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83-Bi-214m

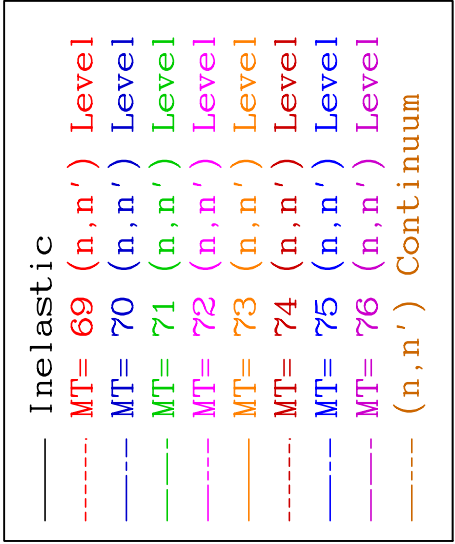
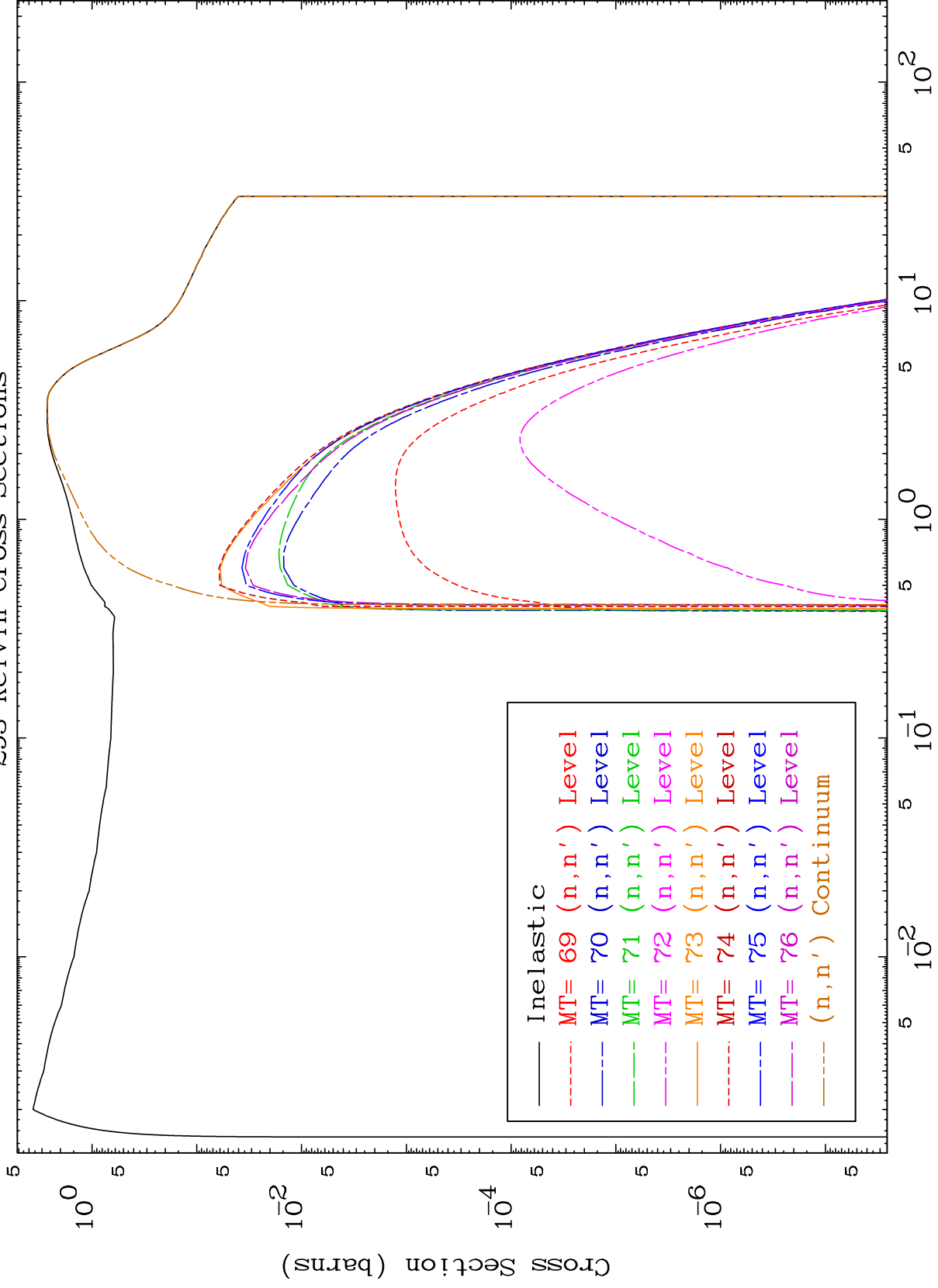
(n,n') Levels  
293 Kelvin Cross Sections



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(n,n') Levels  
293 Kelvin Cross Sections

83-Bi-214m



10

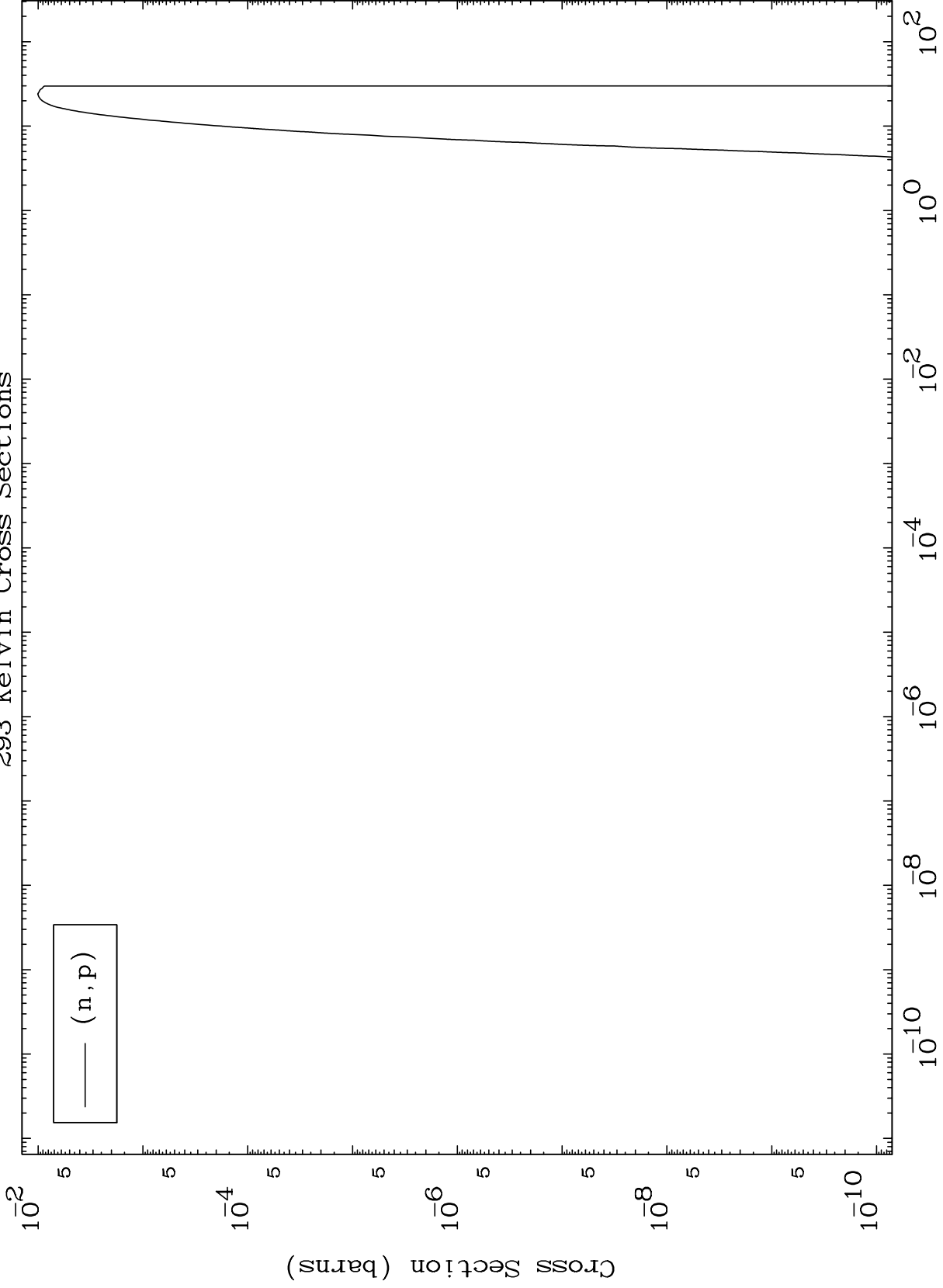
Incident Energy (MeV)

83-Bi-214m

MAT 8341

(n,p) Levels  
293 Kelvin Cross Sections

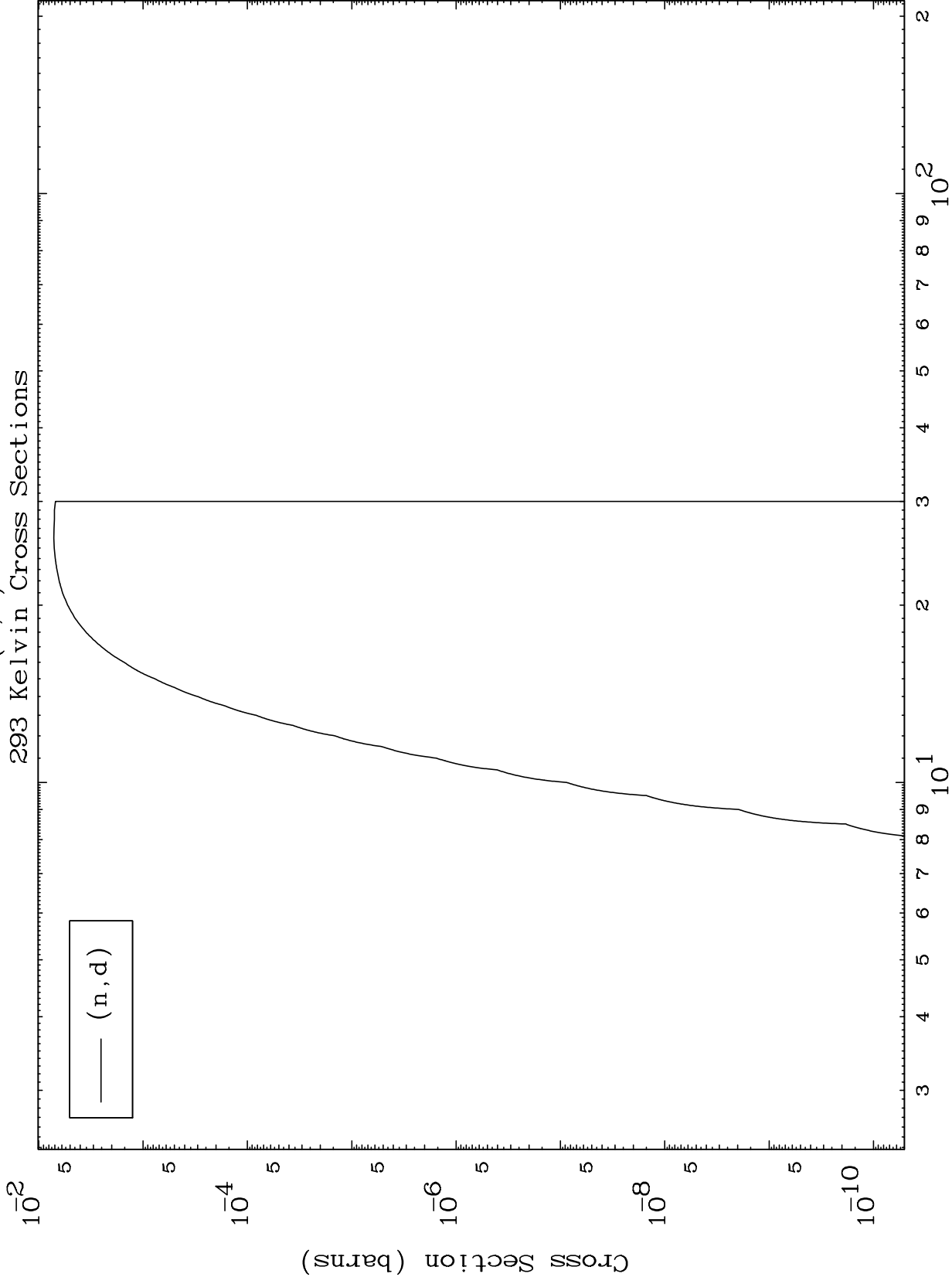
83-Bi-214m



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(n,d) Levels  
293 Kelvin Cross Sections

83-Bi-214m



12

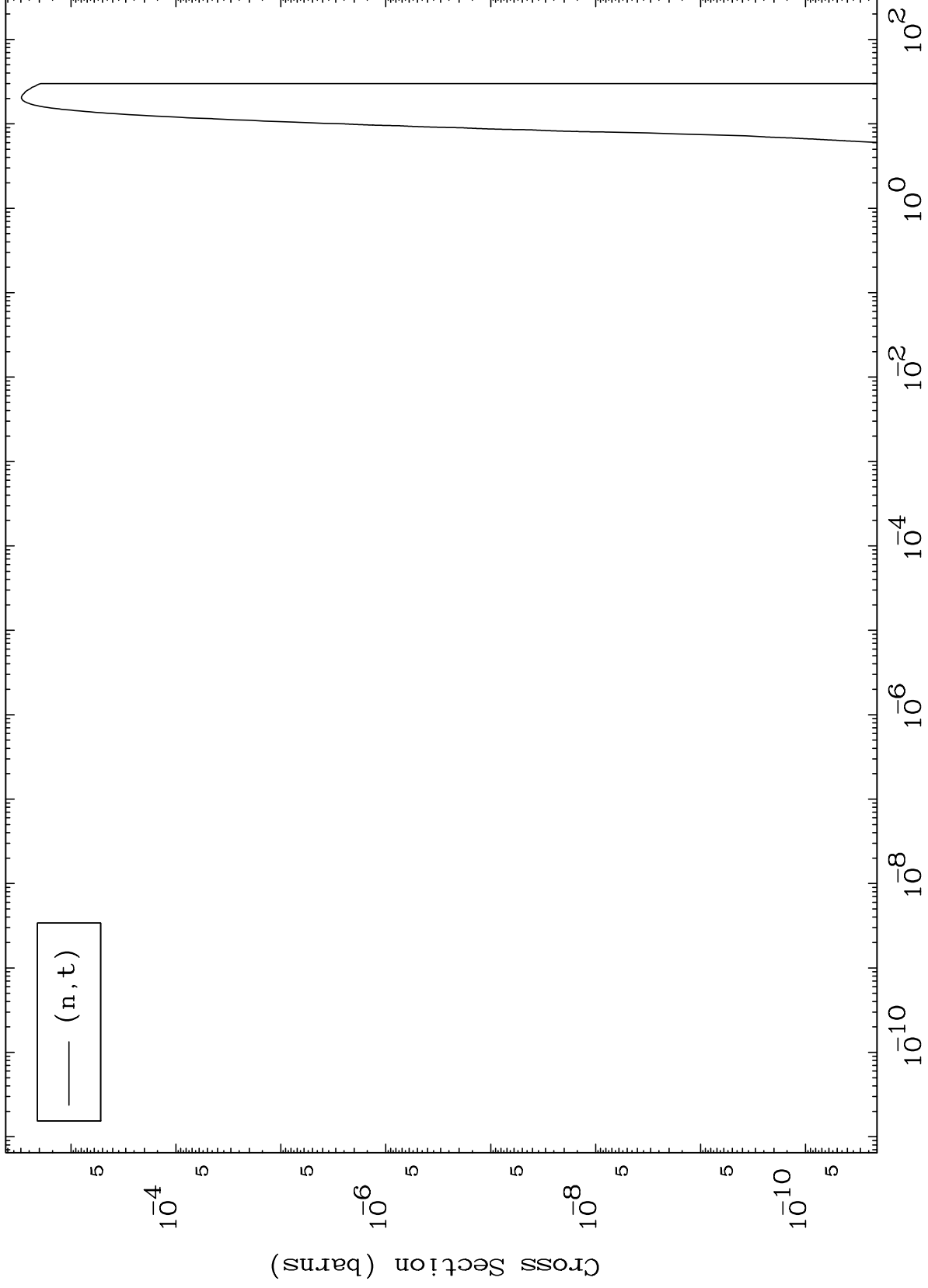
Incident Energy (MeV)

83-Bi-214m

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(n,t) Levels  
293 Kelvin Cross Sections

83-Bi-214m



13

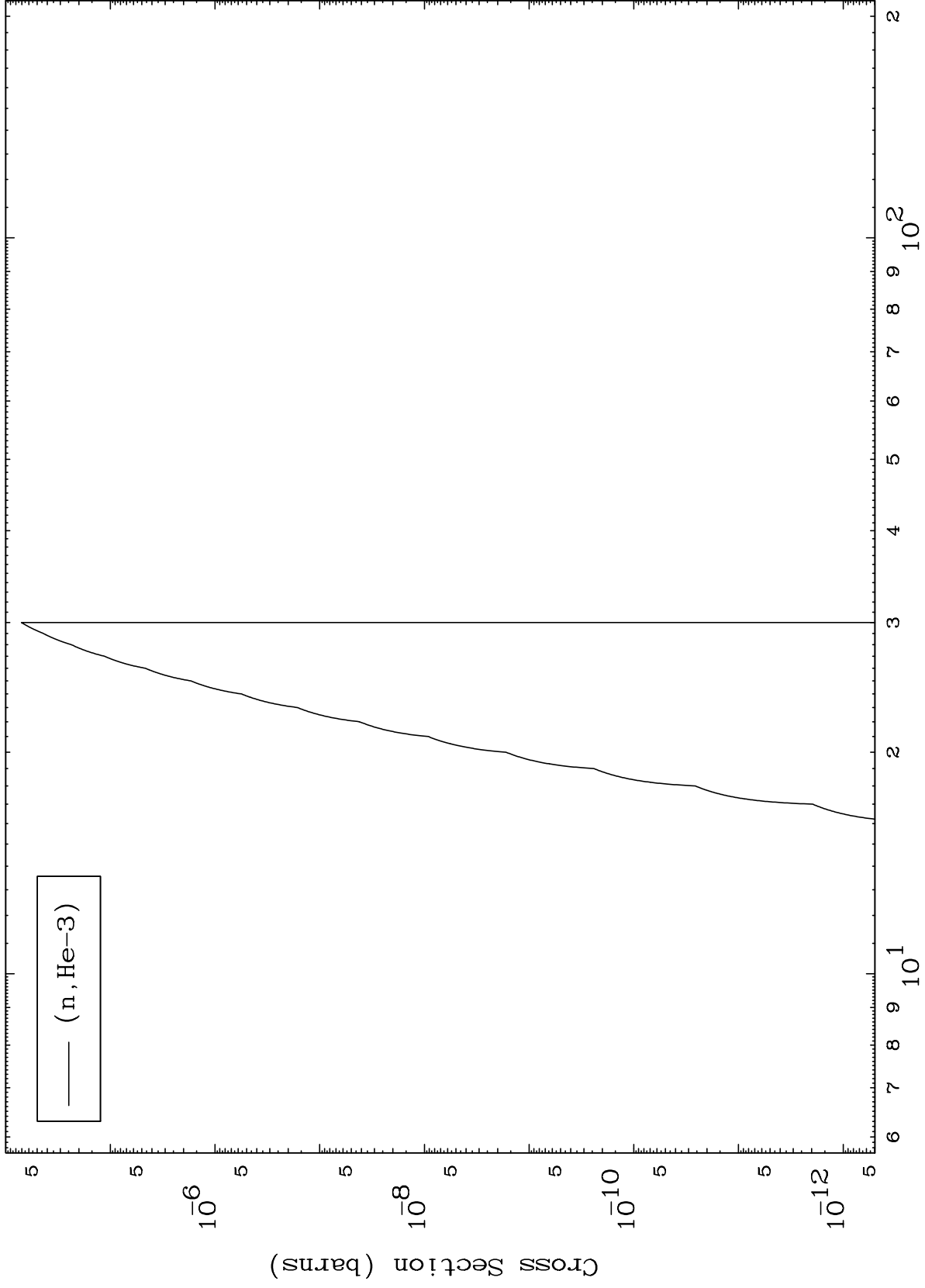
Incident Energy (MeV)

83-Bi-214m

MAT 8341

(n,He3) Levels  
293 Kelvin Cross Sections

83-Bi-214m



14

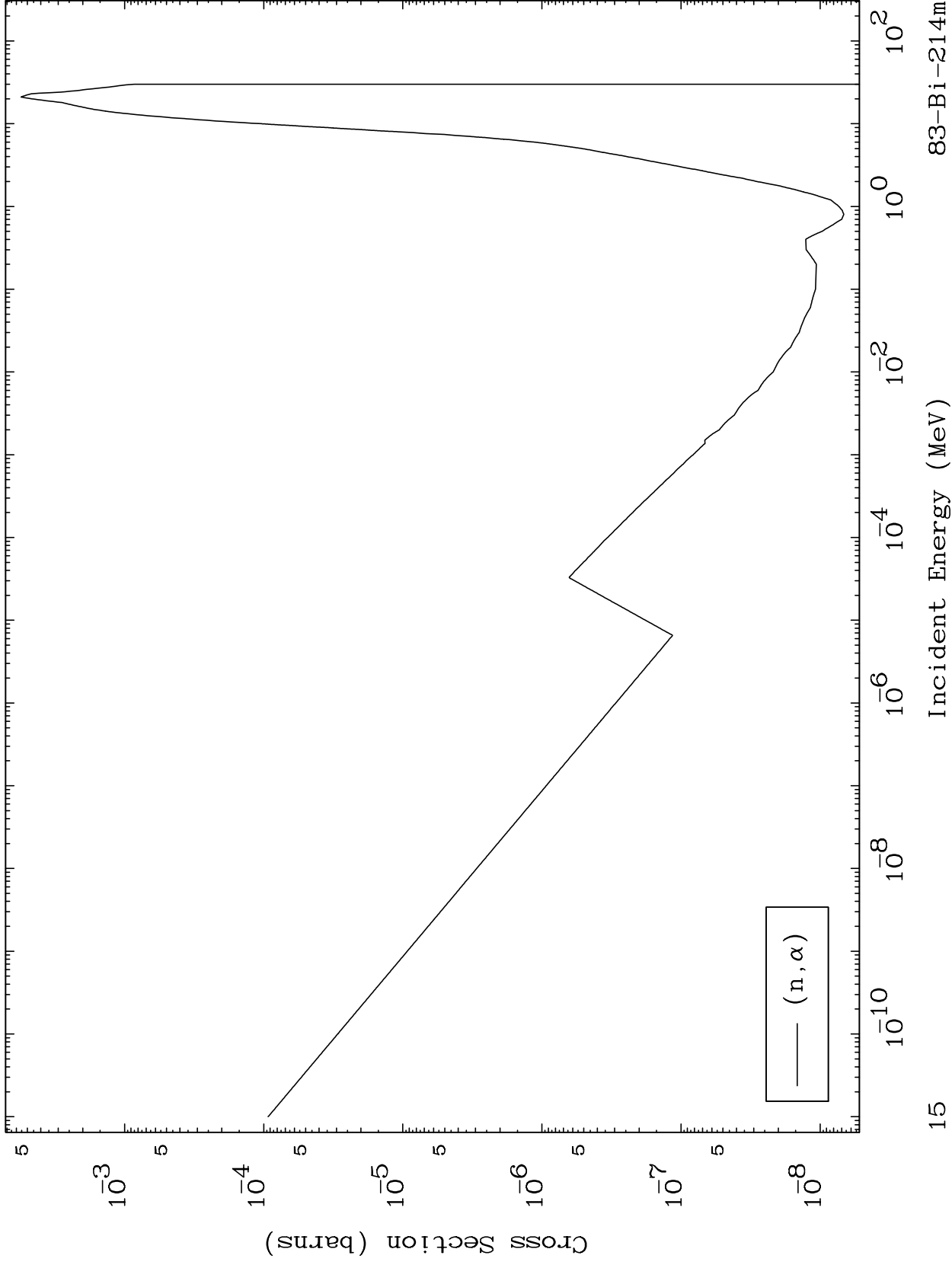
Incident Energy (MeV)

83-Bi-214m

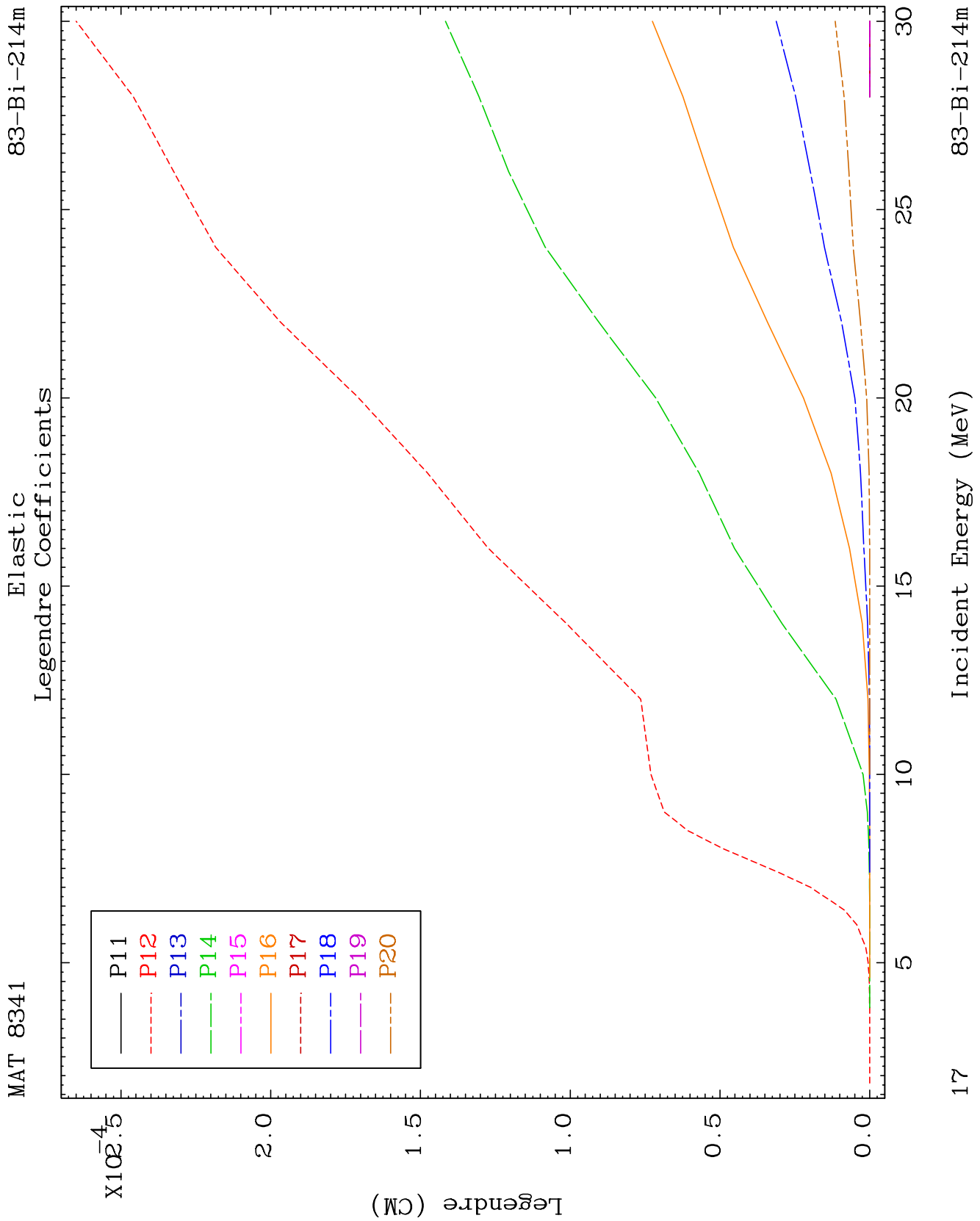
MAT 8341

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

83-Bi-214m



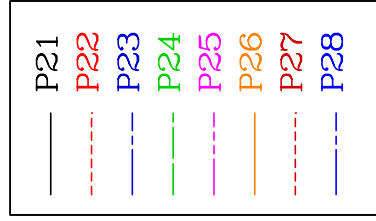




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Elastic  
Legendre Coefficients

83-Bi-214m



$\times 10^{-6}$

Legendre (CM)

3

2

1

0

15

20

25

30

18

Incident Energy (MeV)

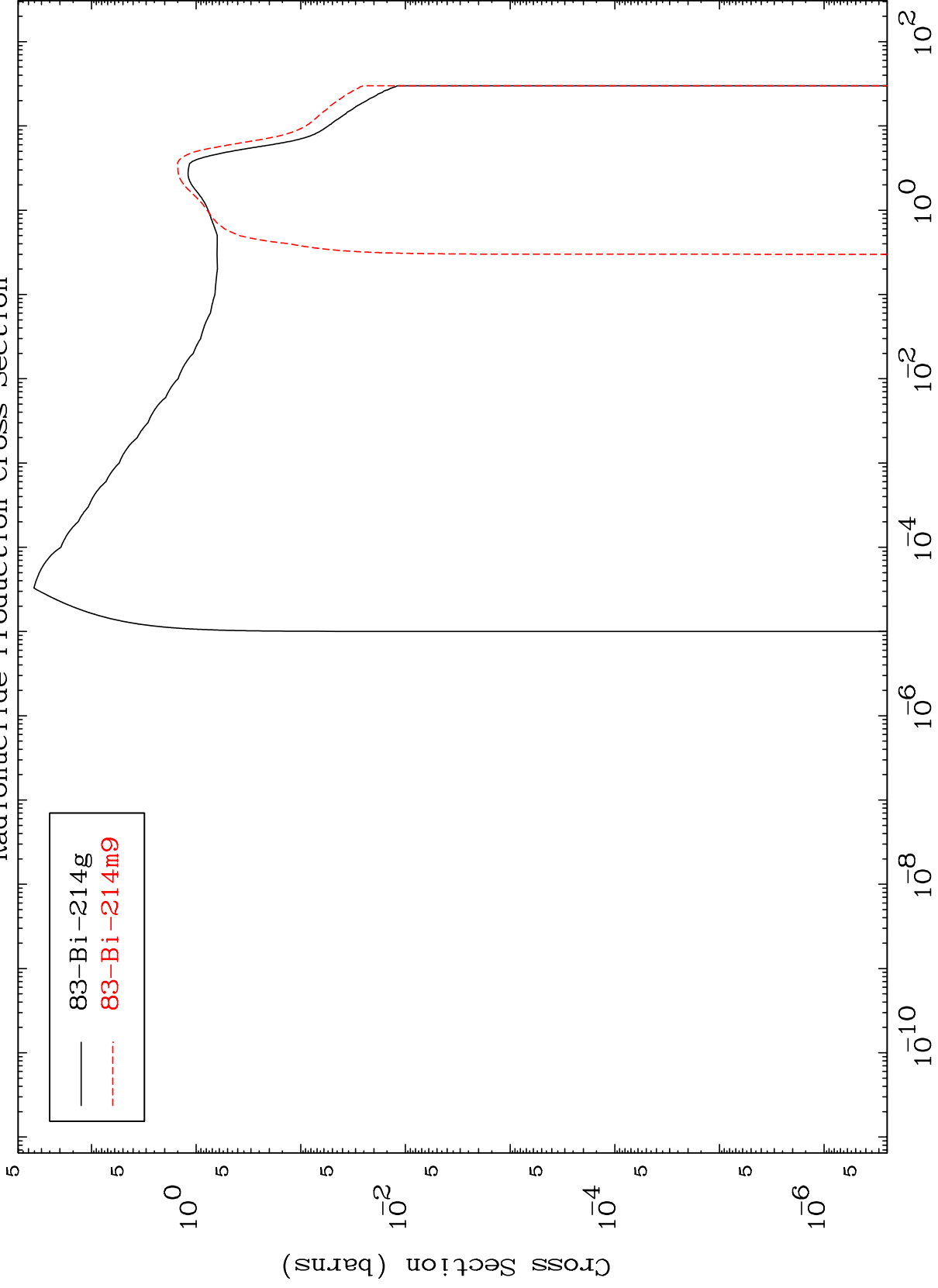
83-Bi-214m



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83-Bi-214m

Inelastic  
Radionuclide Production Cross Section



19

83-Bi-214m

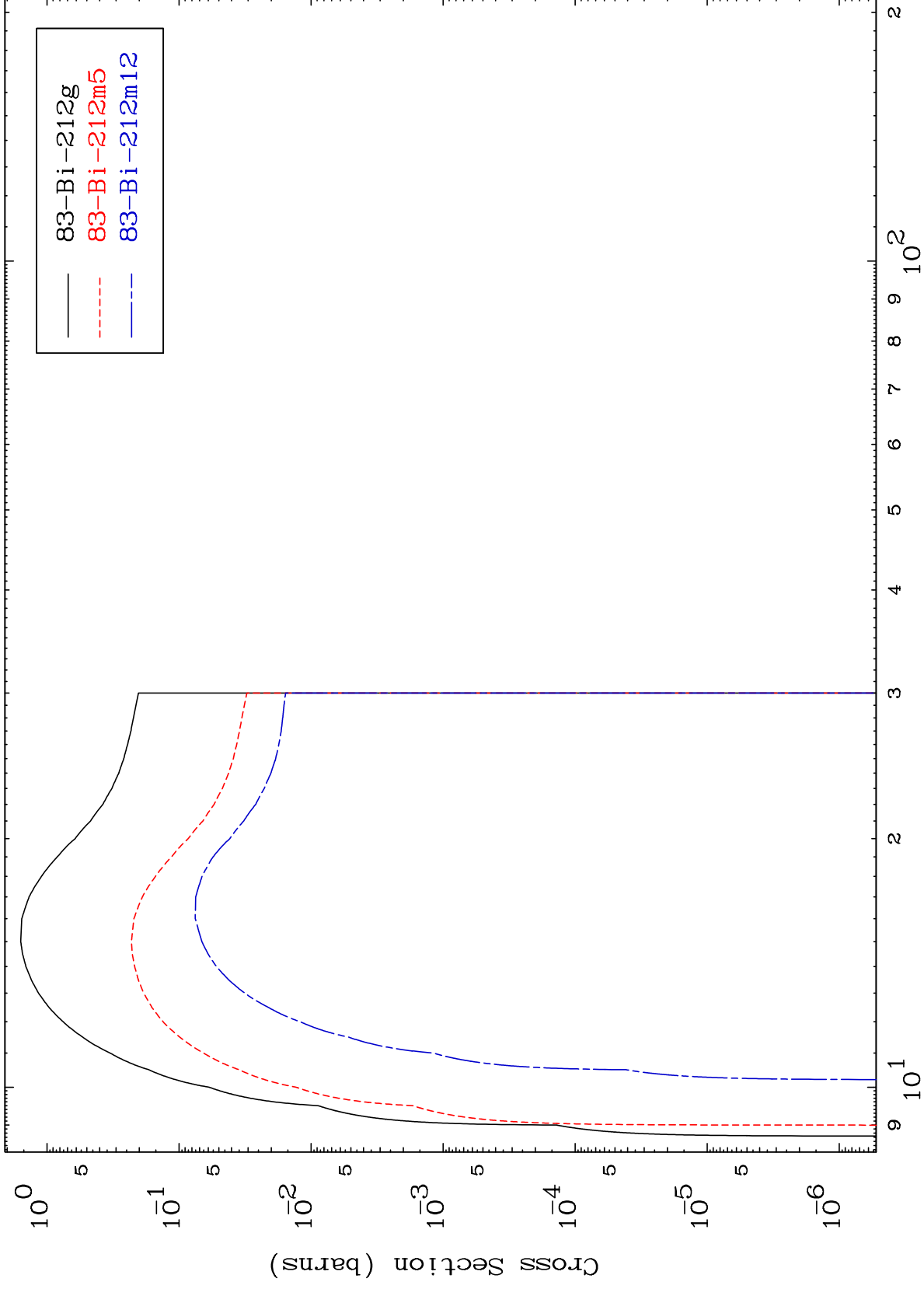
Incident Energy (MeV)

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(n,3n)

83-Bi-214m

Radionuclide Production Cross Section



Incident Energy (MeV)

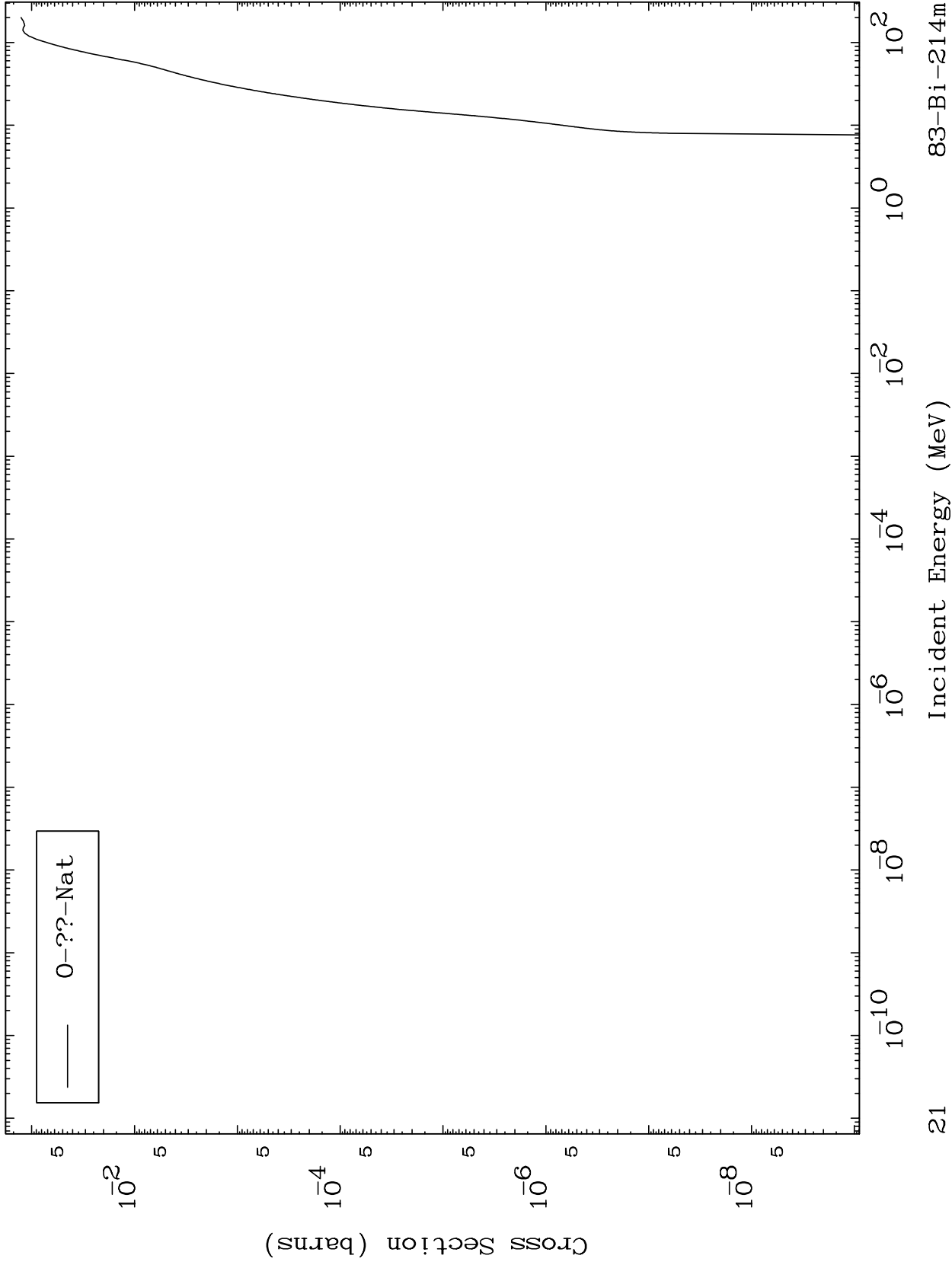
83-Bi-214m

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Fission

<sup>83</sup>Bi-214m

Radionuclide Production Cross Section



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$^{83}\text{Bi}-214\text{m}$

$(n, \gamma)$   
Radionuclide Production Cross Section

