

Program EVALPLOT
(Version 2021-1)

by

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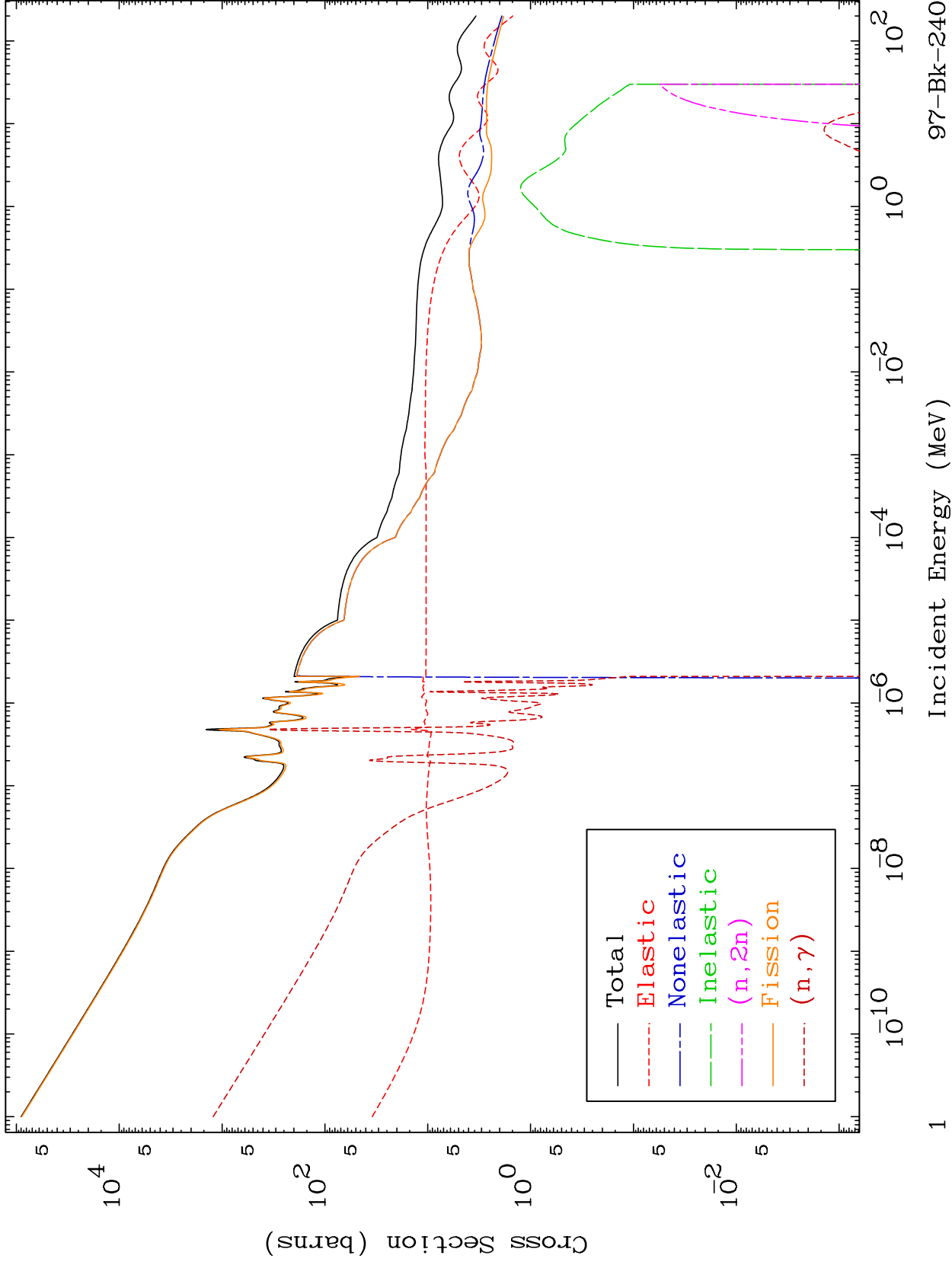
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Press Mouse Button to Start

MAT 9725

Neutron Major
293 Kelvin Cross Sections

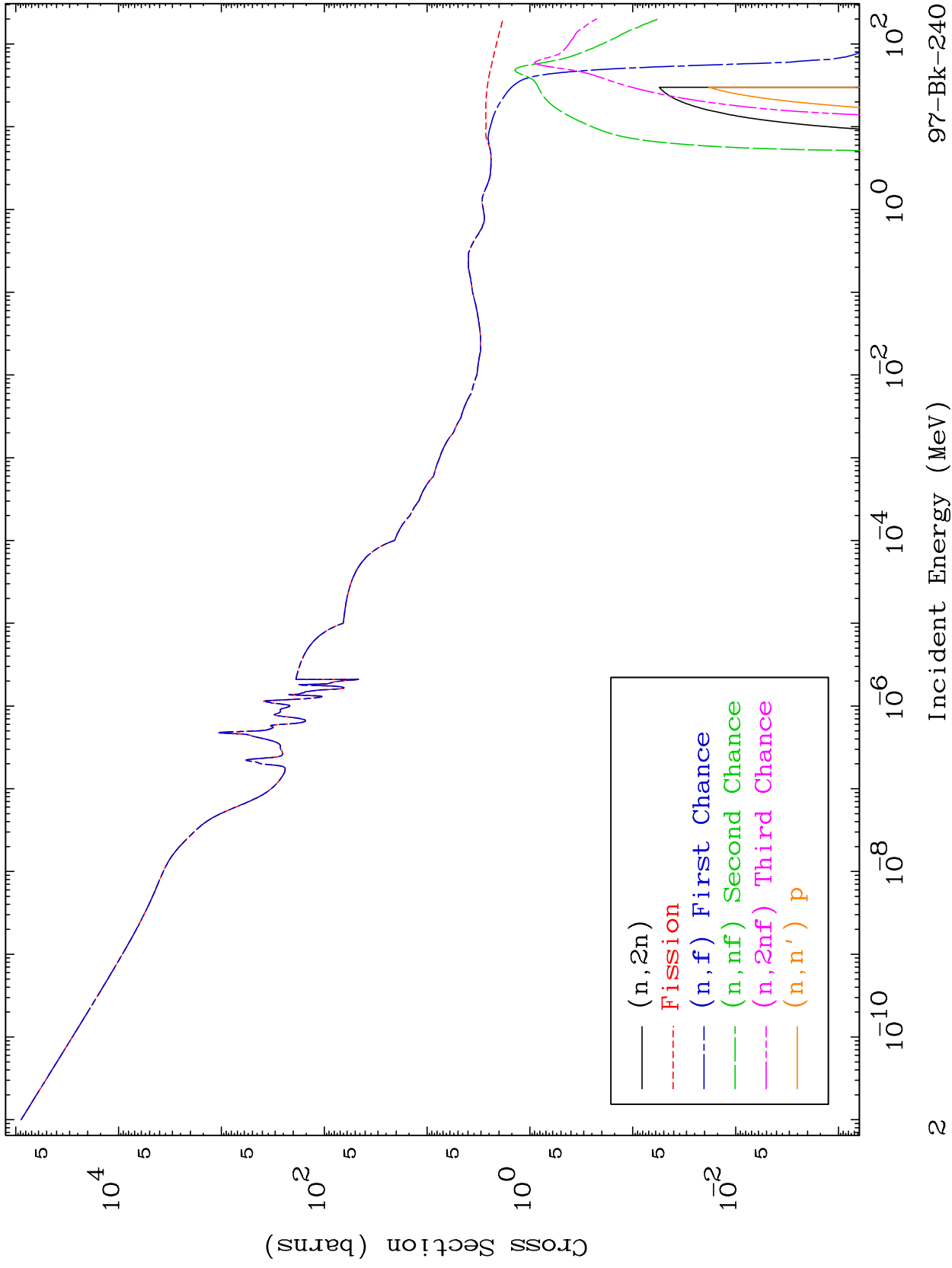
97-Bk-240



MAT 9725

Neutron Absorption
293 Kelvin Cross Sections

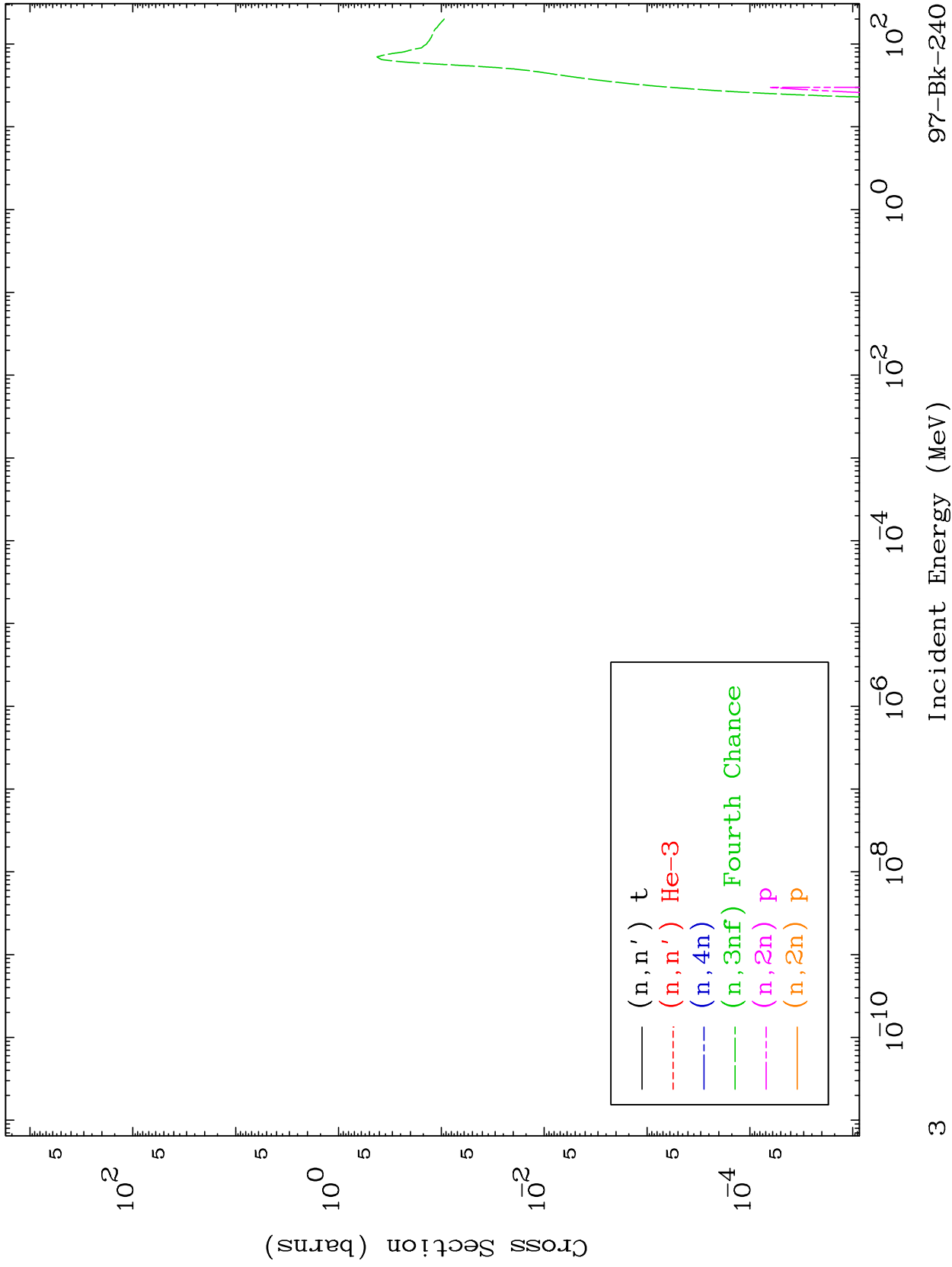
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MAT 9725

Neutron Absorption
293 Kelvin Cross Sections

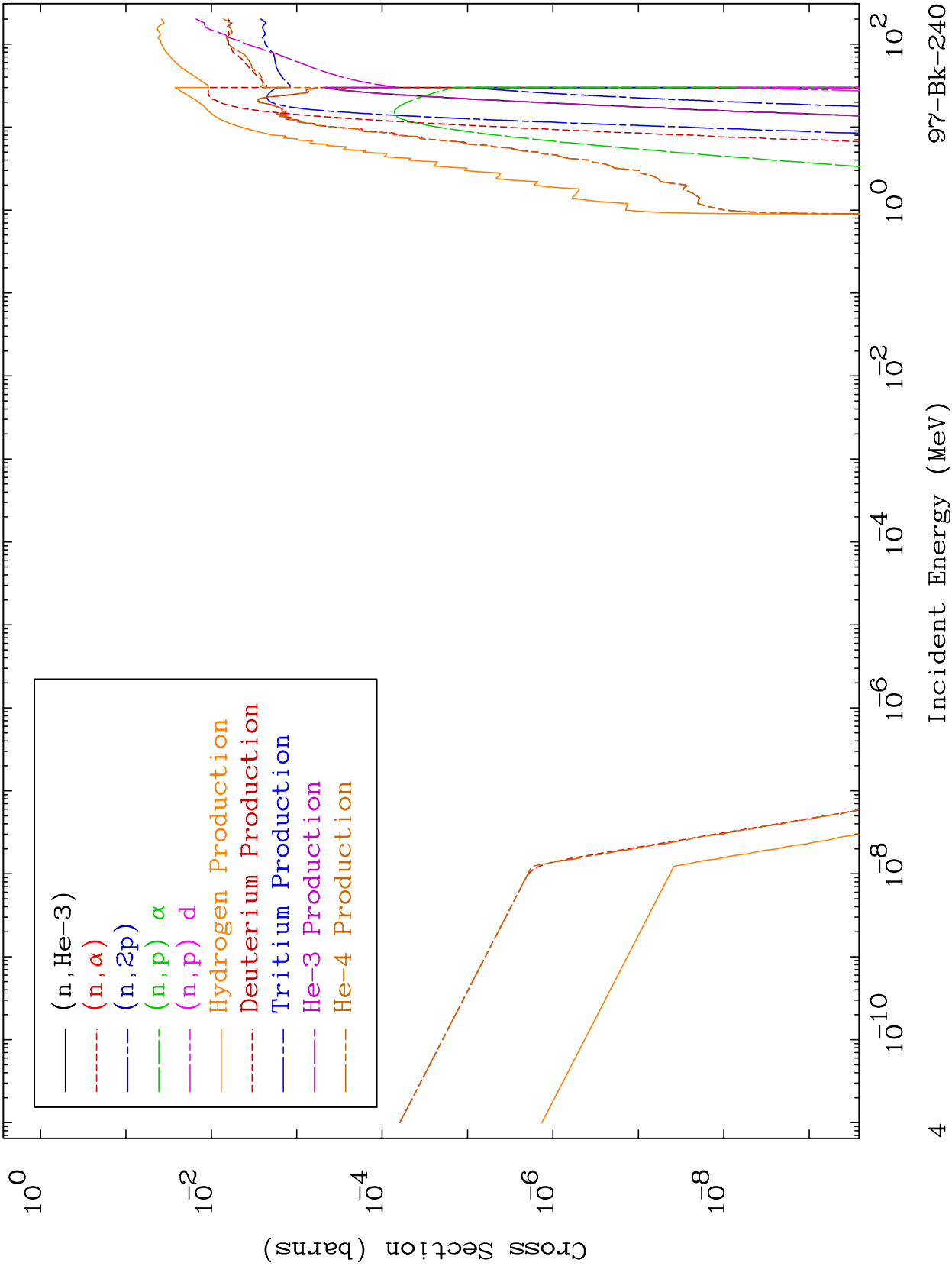
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MAT 9725

Neutron Absorption
293 Kelvin Cross Sections

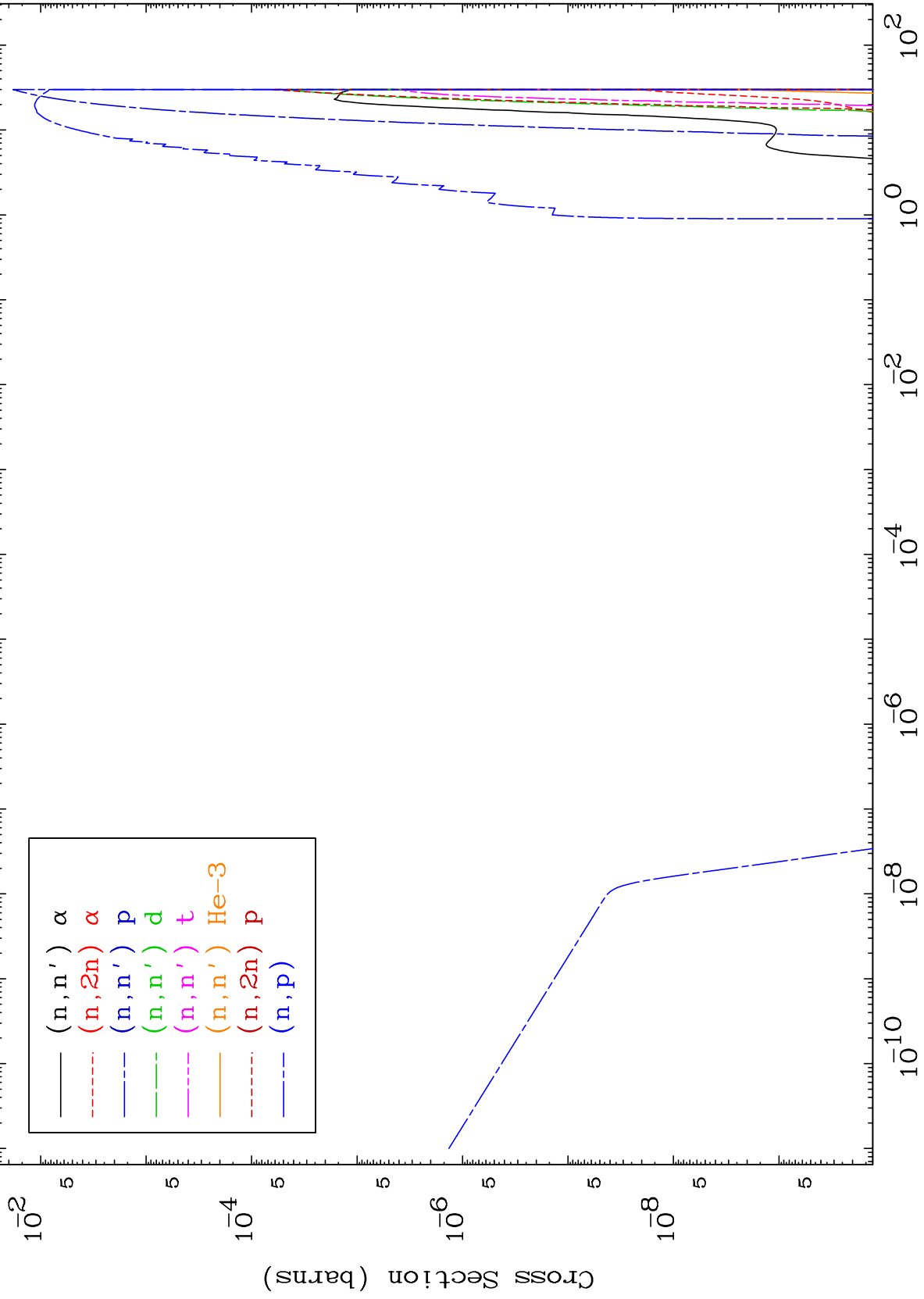
97-Bk-240



MAT 9725

Charged Particle
293 Kelvin Cross Sections

97-Bk-240



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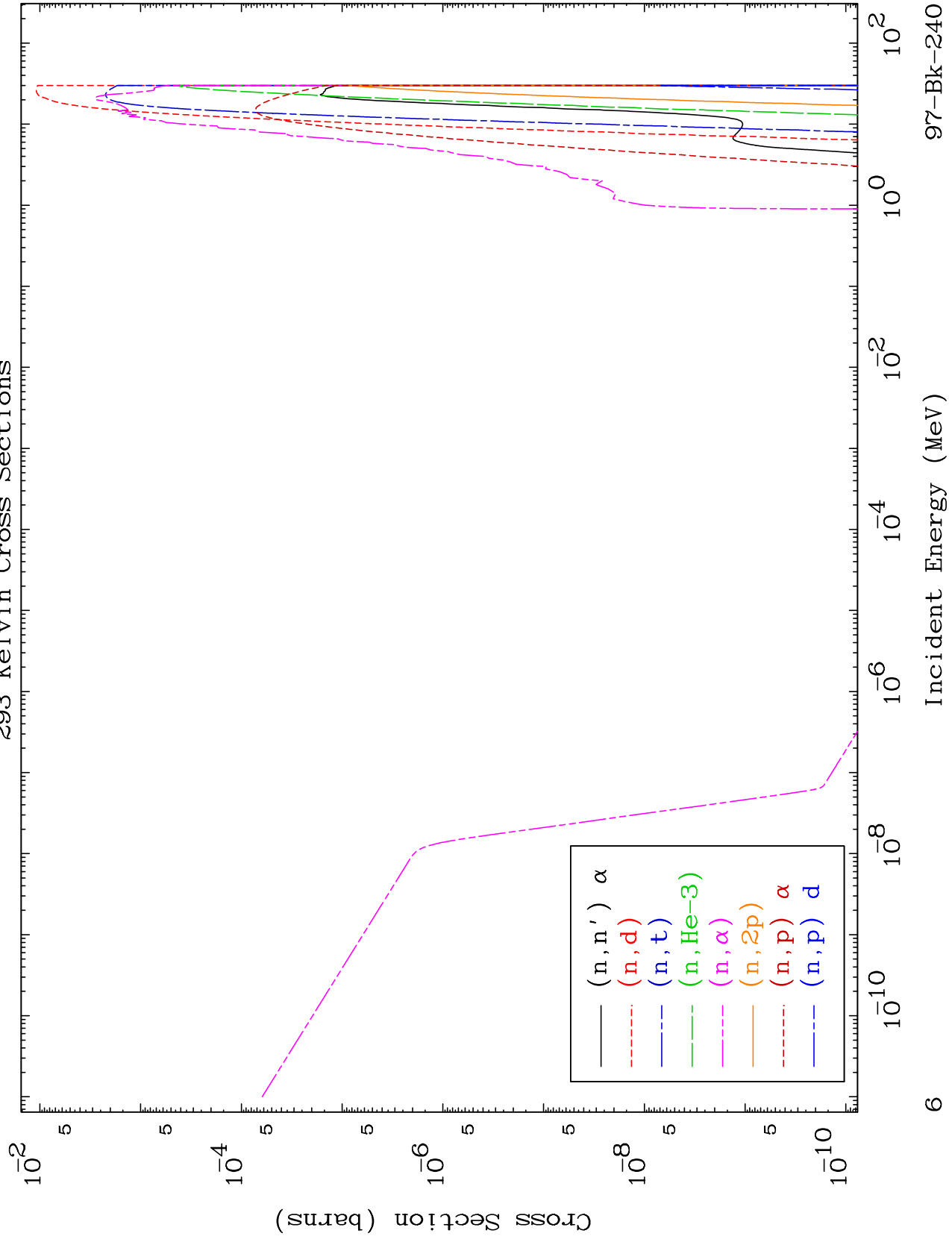
Incident Energy (MeV)

97-Bk-240

MAT 9725

Charged Particle
293 Kelvin Cross Sections

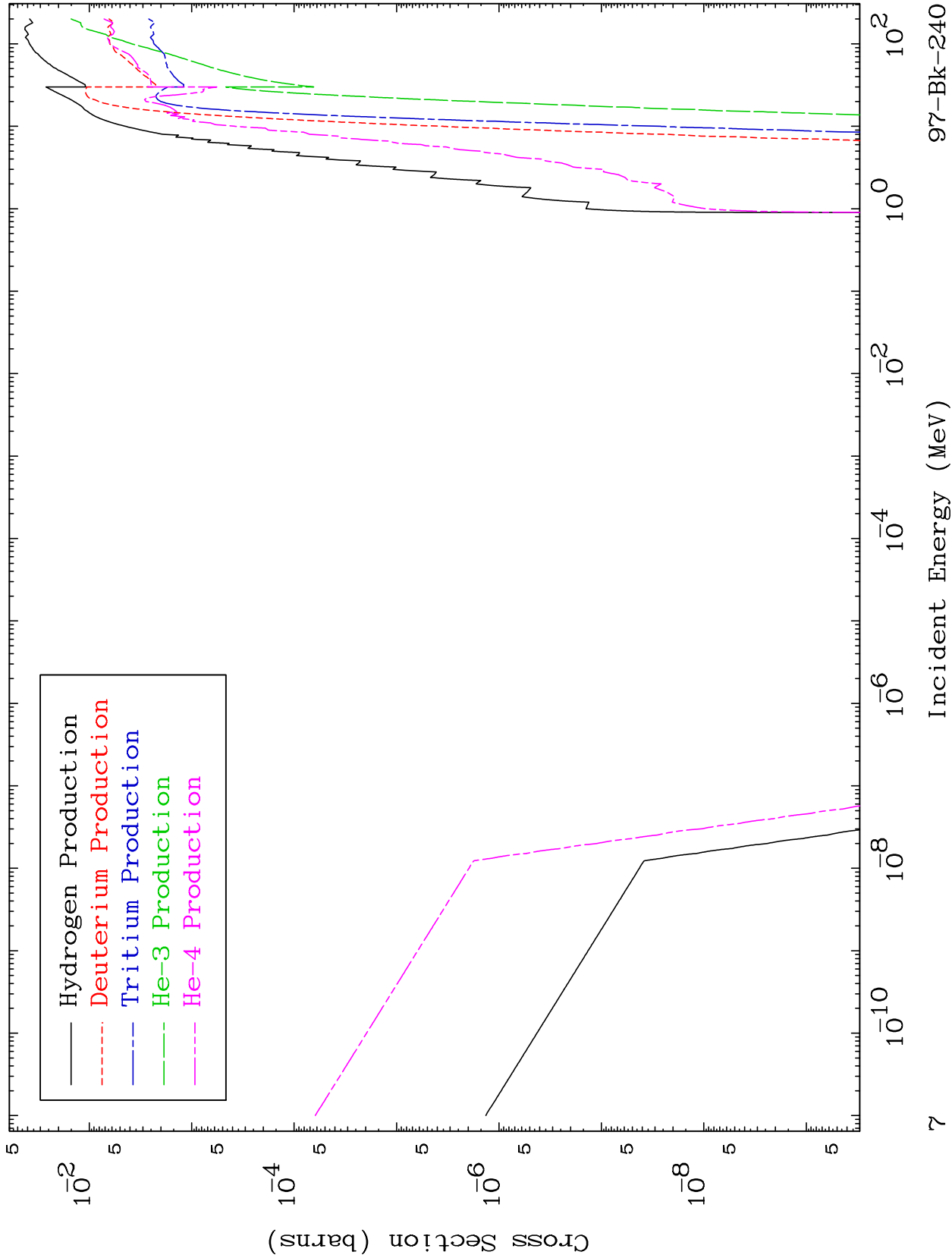
97-Bk-240



MAT 9725

Particle Production
293 Kelvin Cross Sections

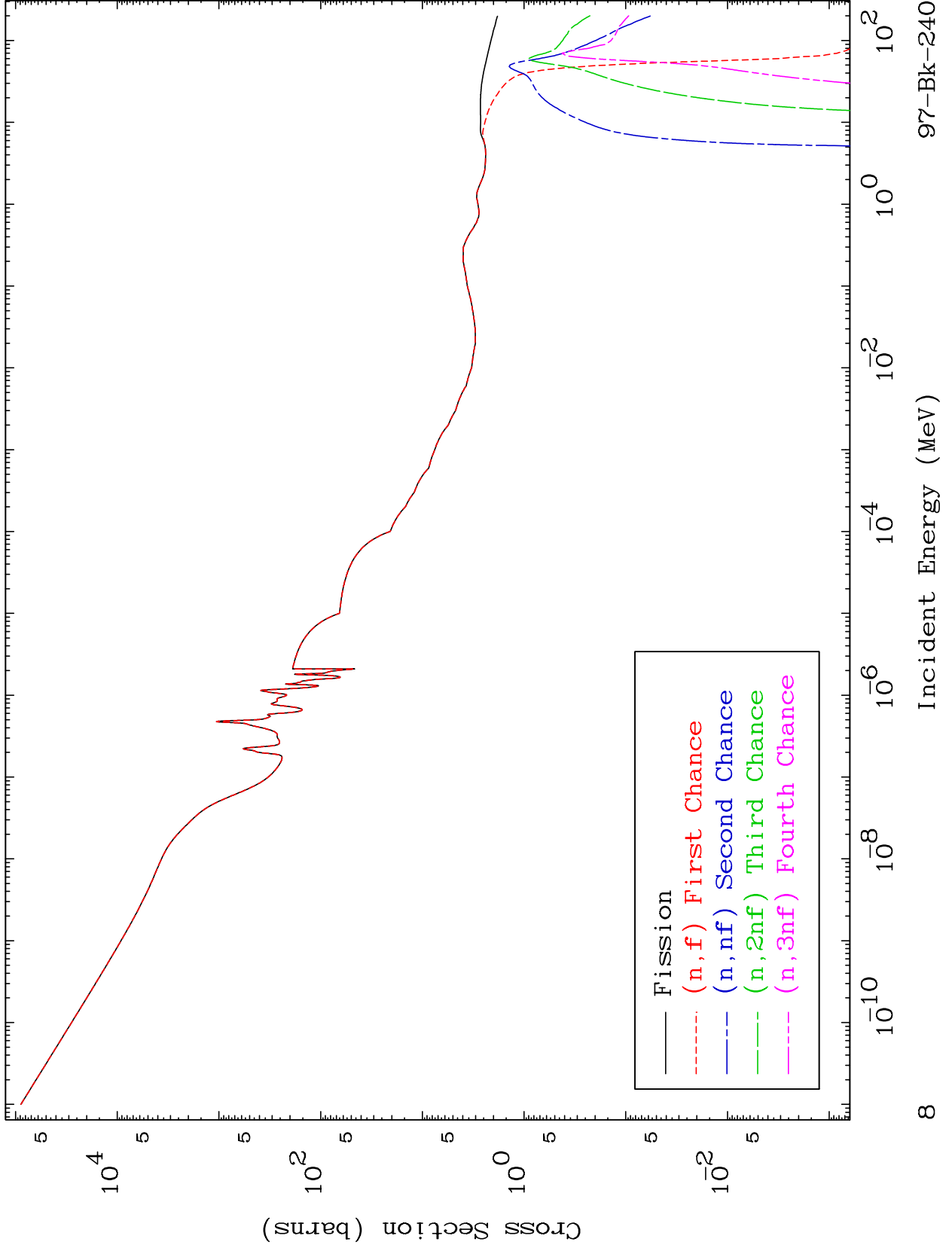
97-Bk-240



MAT 9725

Fission
293 Kelvin Cross Sections

97-Bk-240

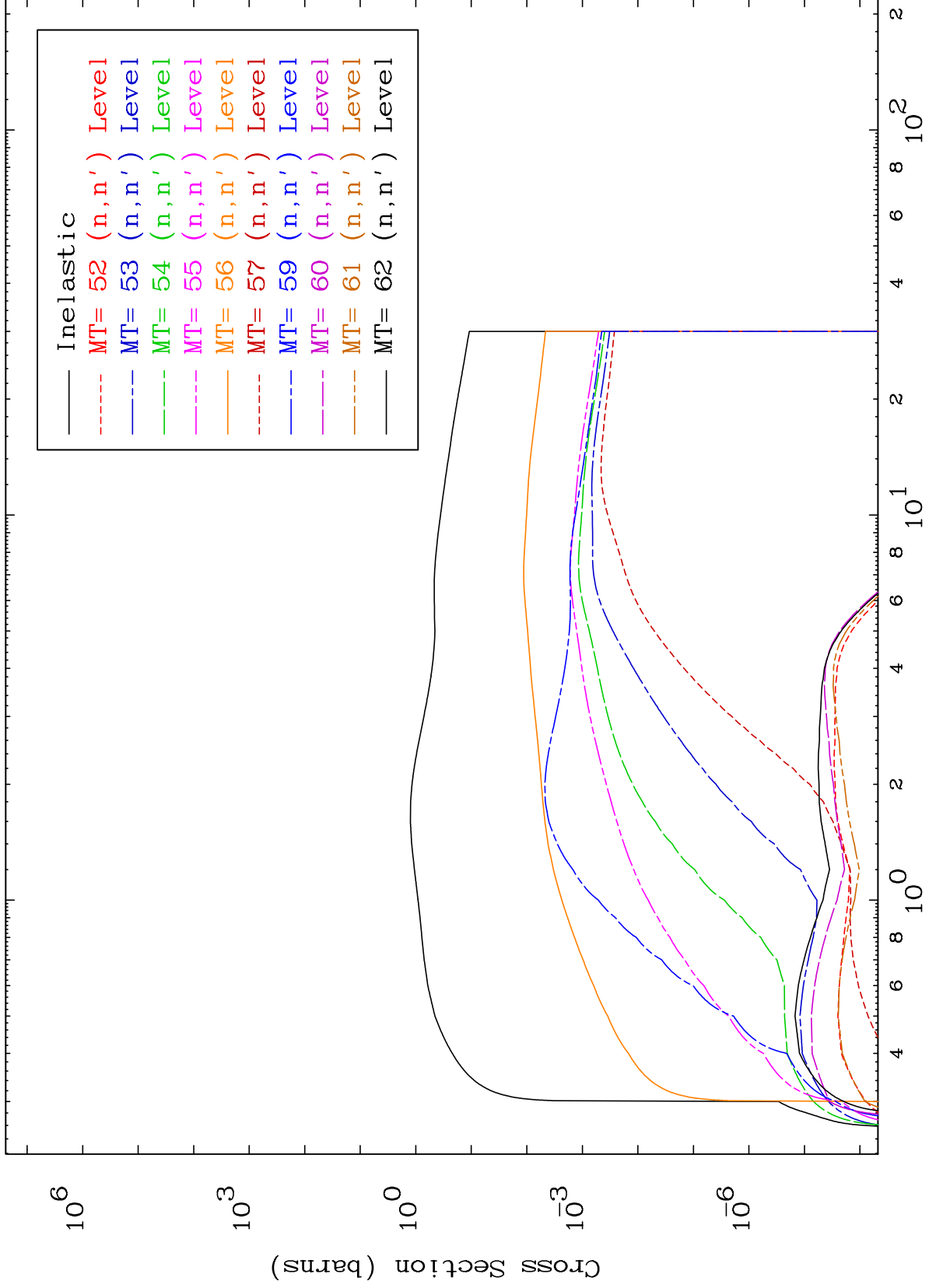


MAT 9725

(n,n') Levels

97-Bk-240

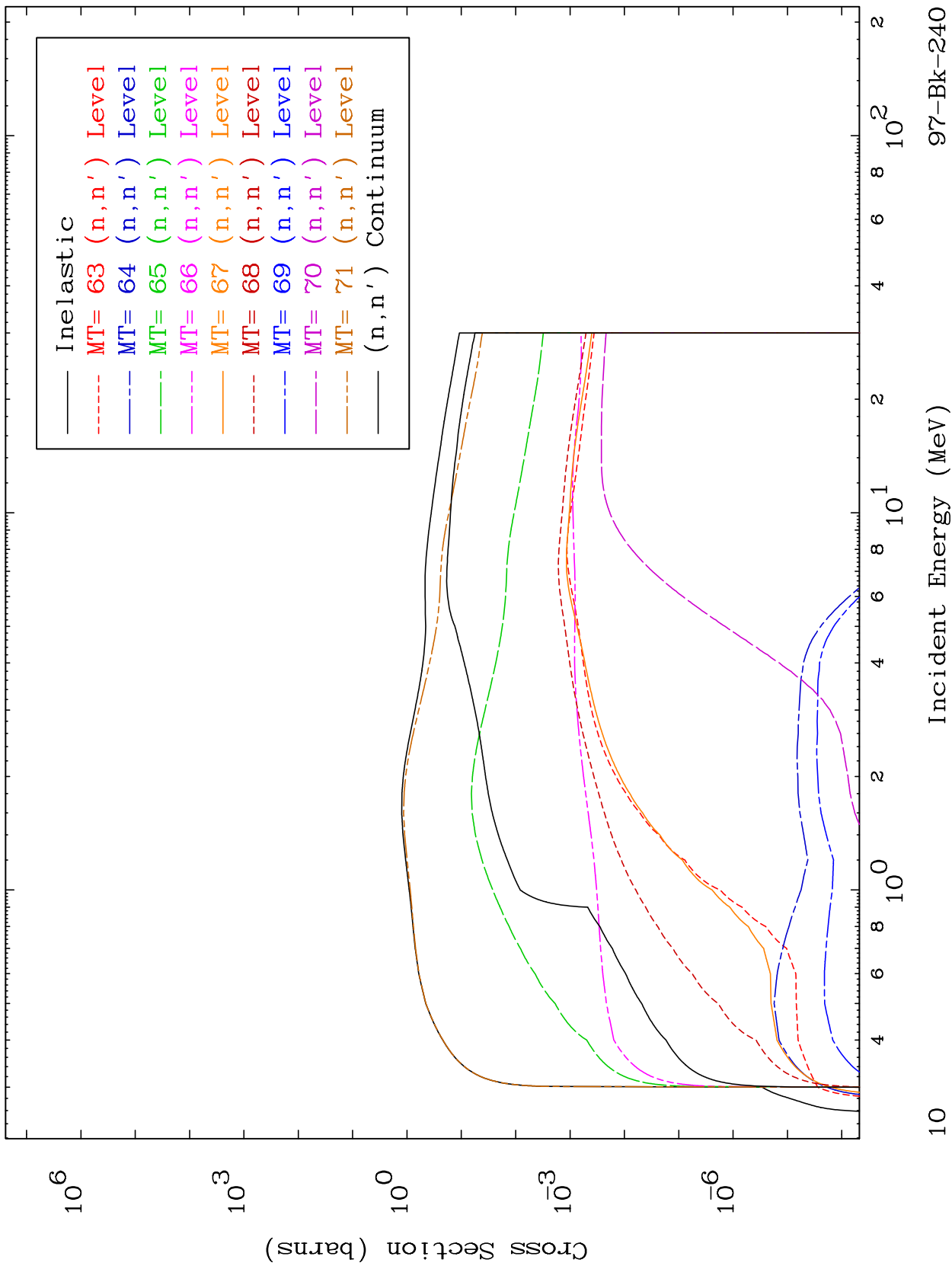
293 Kelvin Cross Sections



MAT 9725

(n,n') Levels
293 Kelvin Cross Sections

97-Bk-240



10

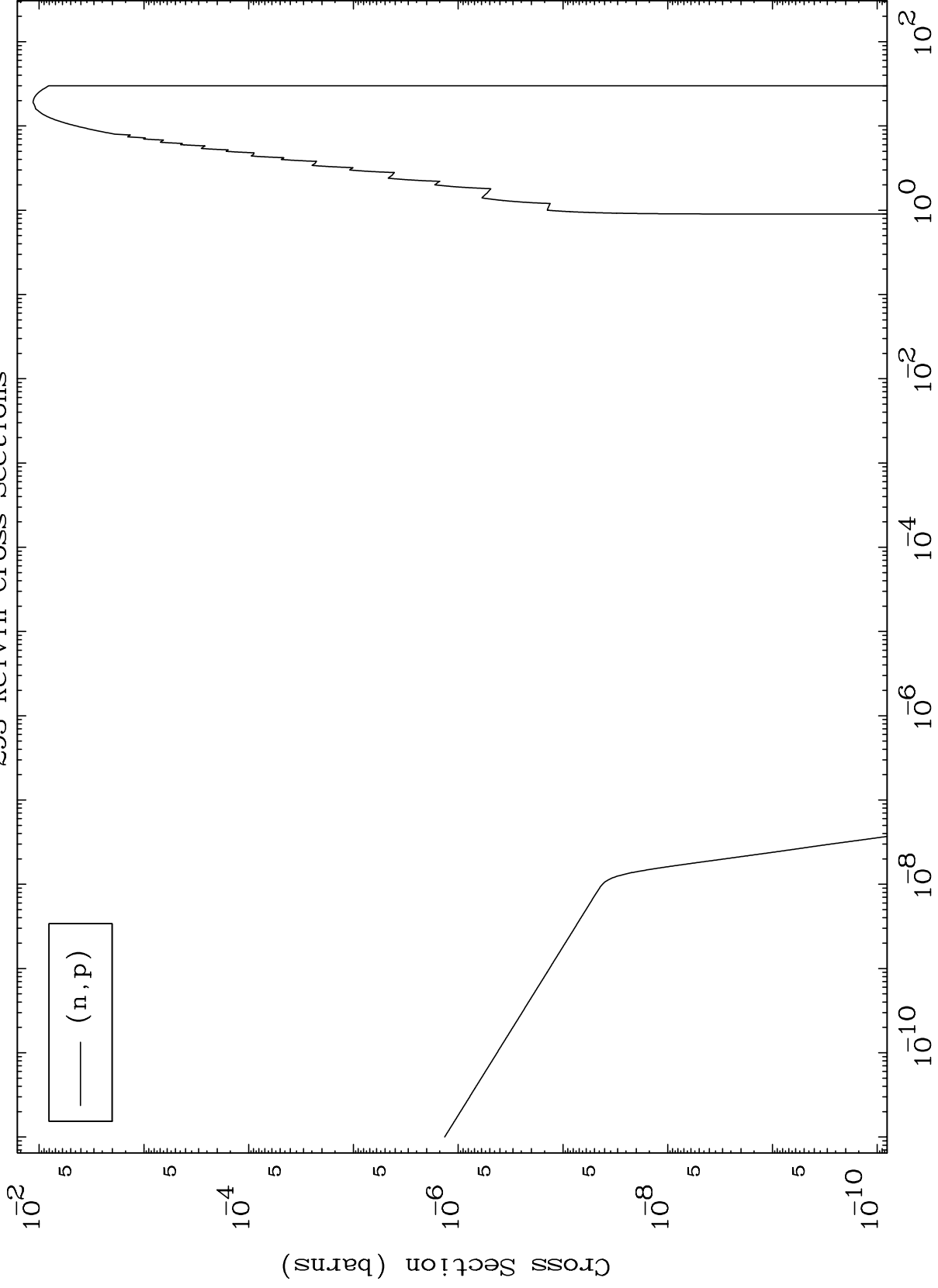
Incident Energy (MeV)

97-Bk-240

MAT 9725

(n,p) Levels
293 Kelvin Cross Sections

97-Bk-240

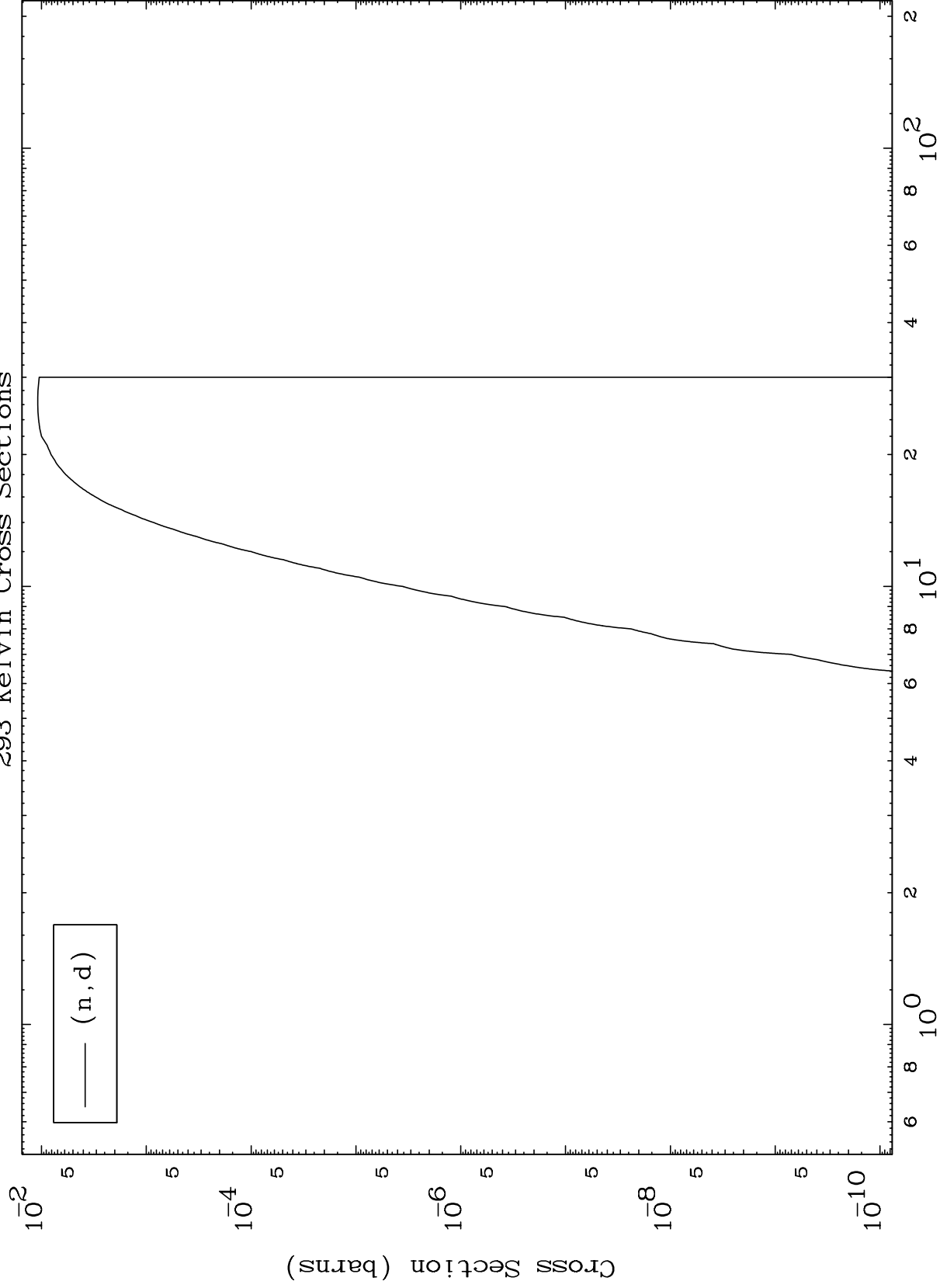


MAT 9725

(n,d) Levels

97-Bk-240

293 Kelvin Cross Sections



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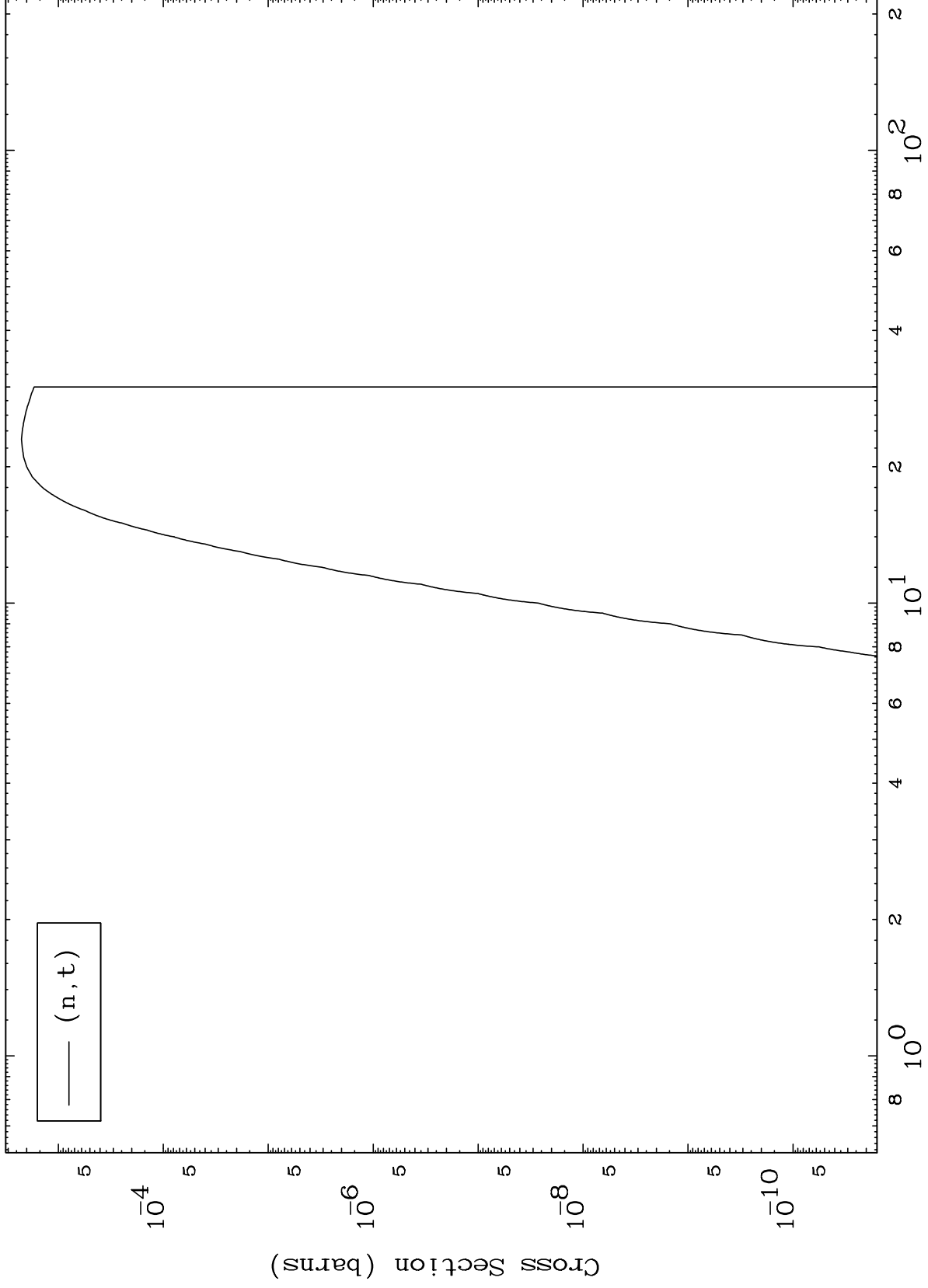
Incident Energy (MeV)

97-Bk-240

MAT 9725

(n,t) Levels
293 Kelvin Cross Sections

97-Bk-240



13

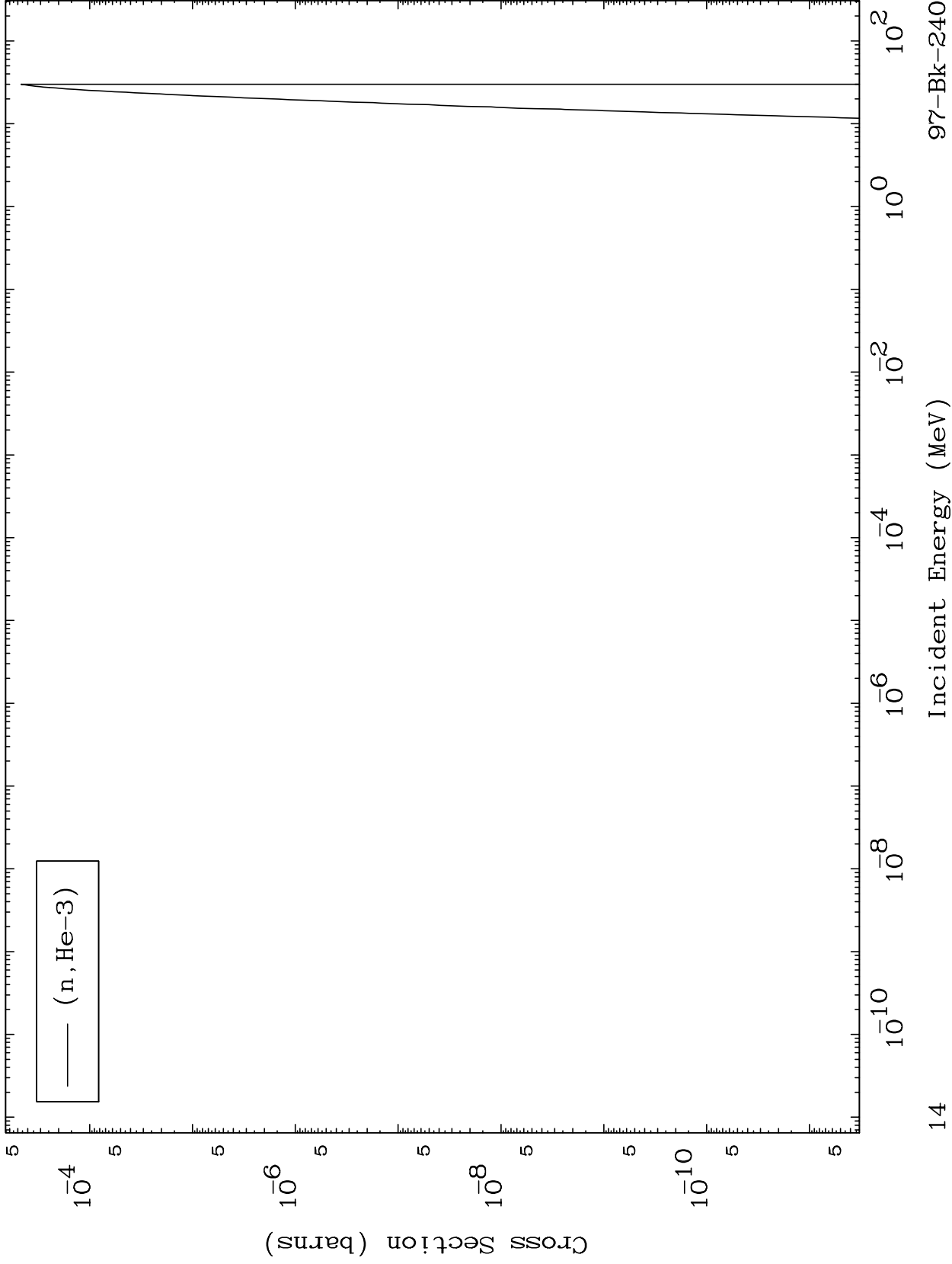
Incident Energy (MeV)

97-Bk-240

MAT 9725

(n,He3) Levels
293 Kelvin Cross Sections

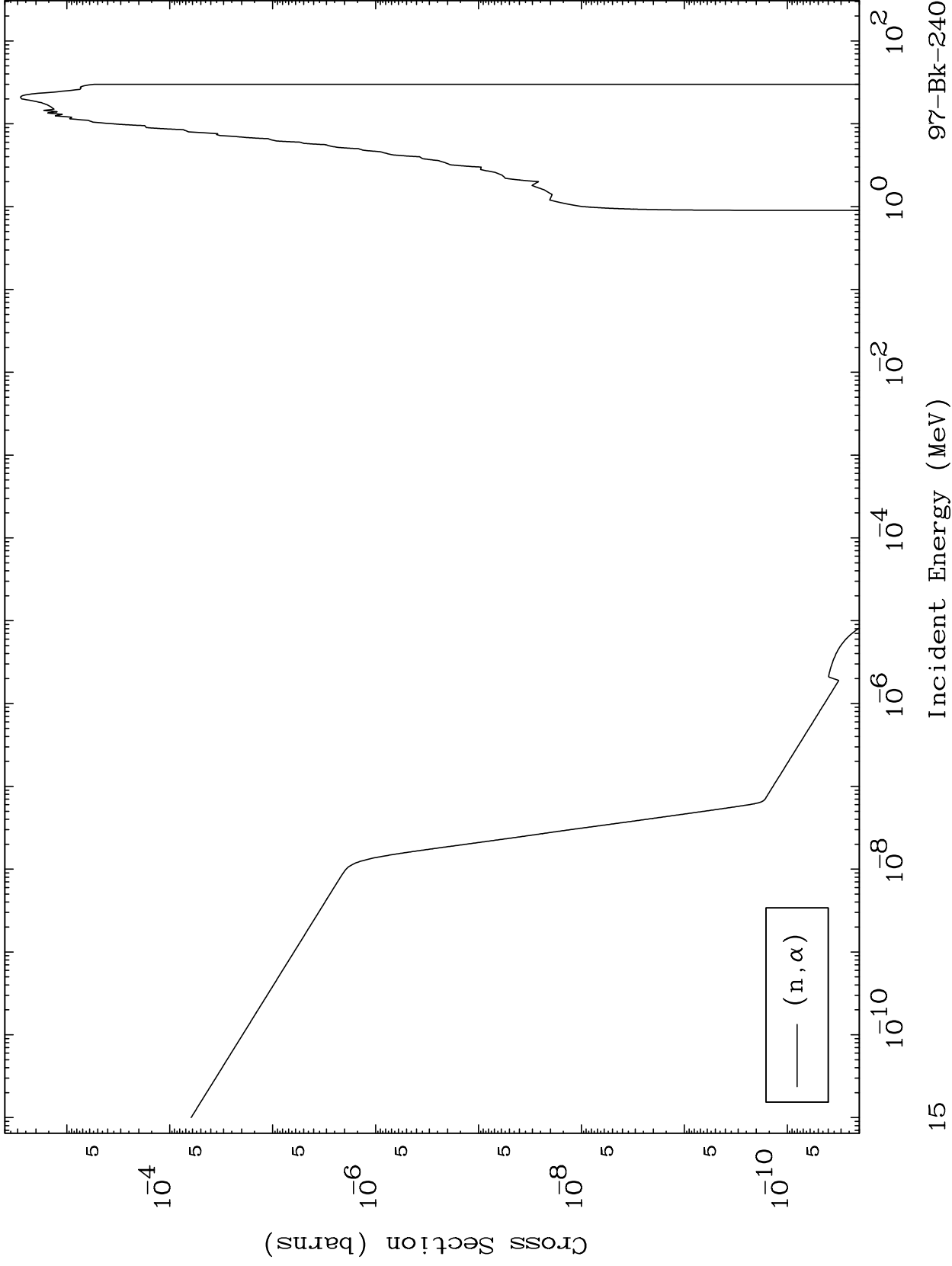
97-Bk-240



MAT 9725

(n,α) Levels
293 Kelvin Cross Sections

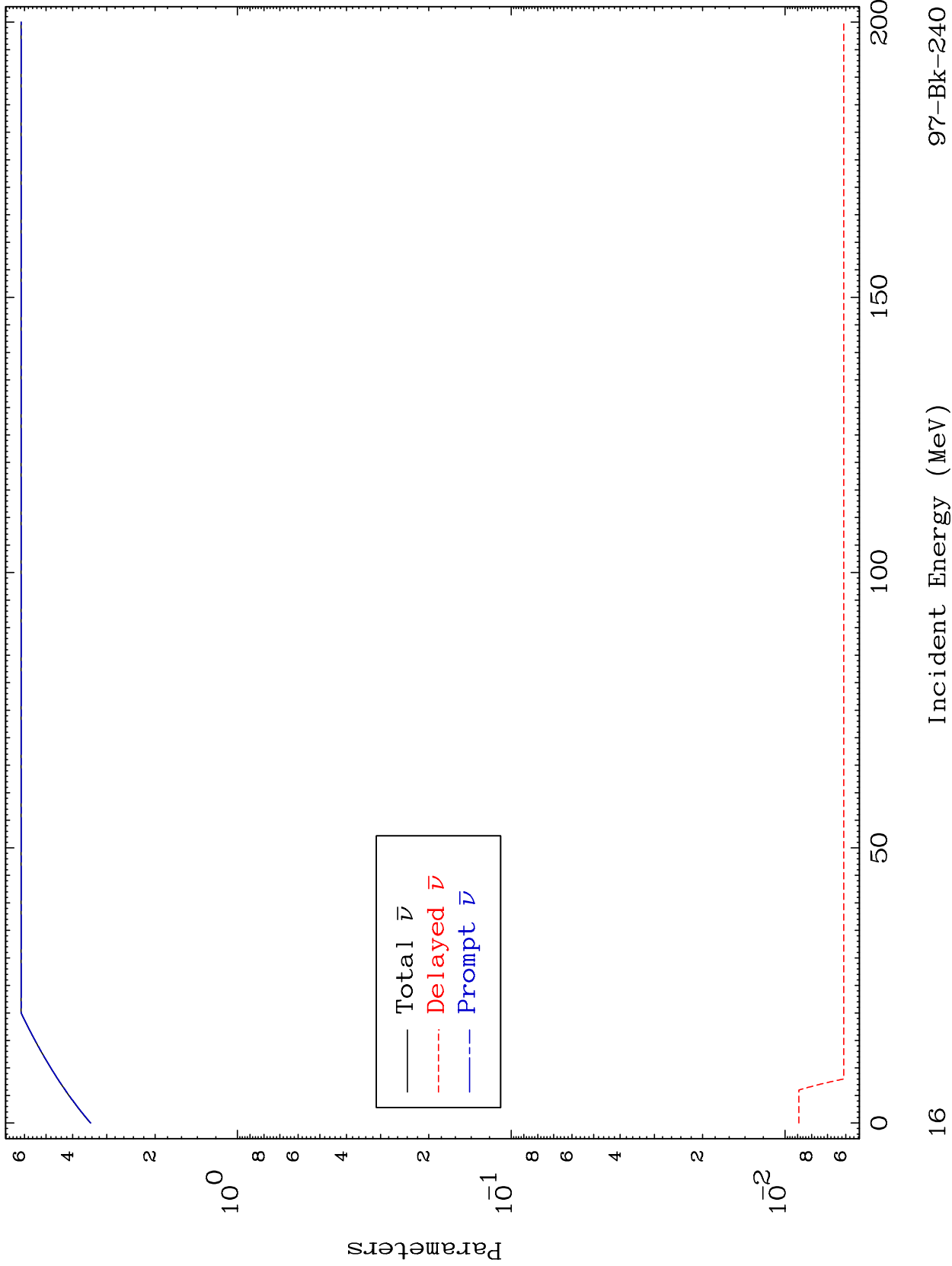
97-Bk-240



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Energy Release
Parameters

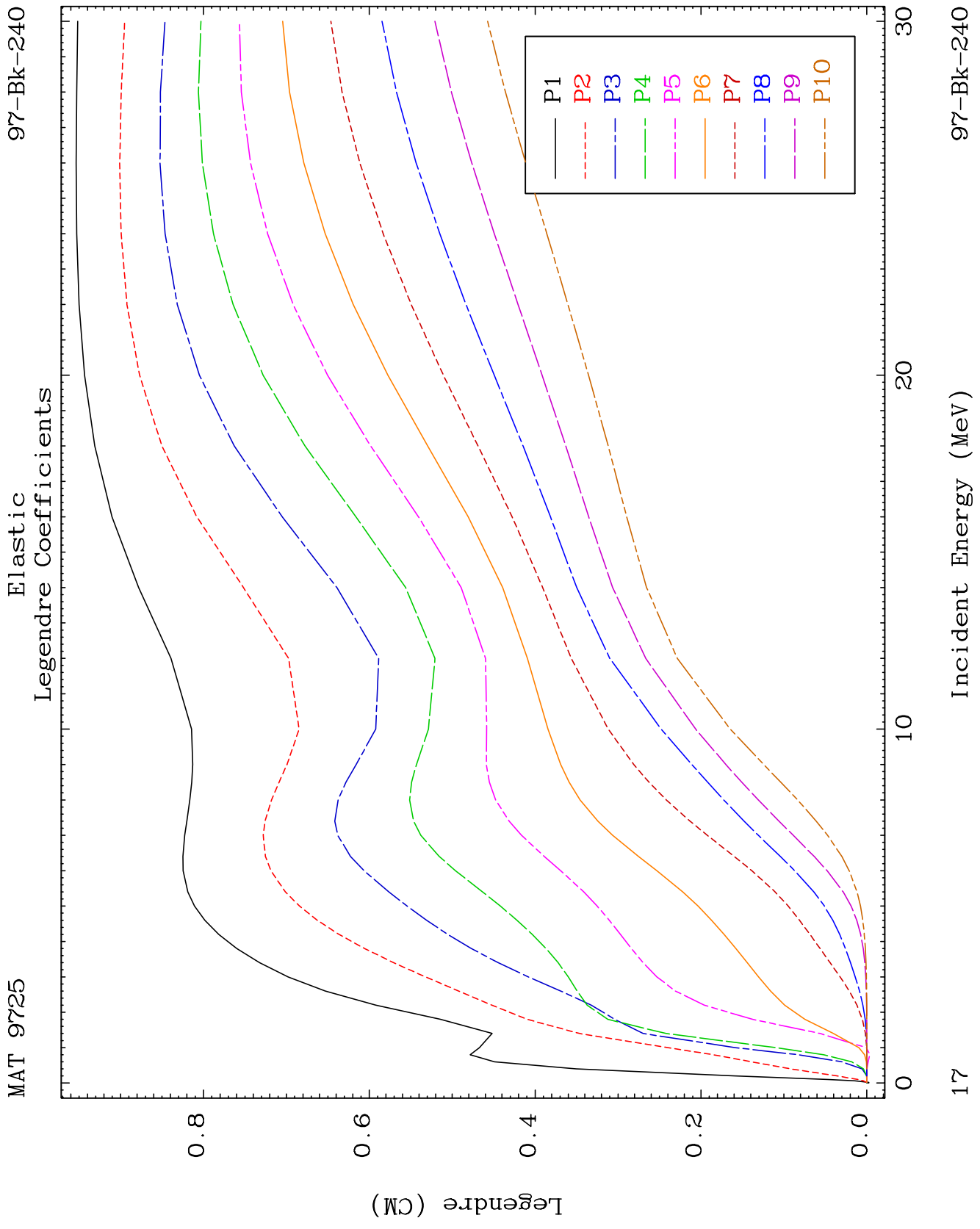
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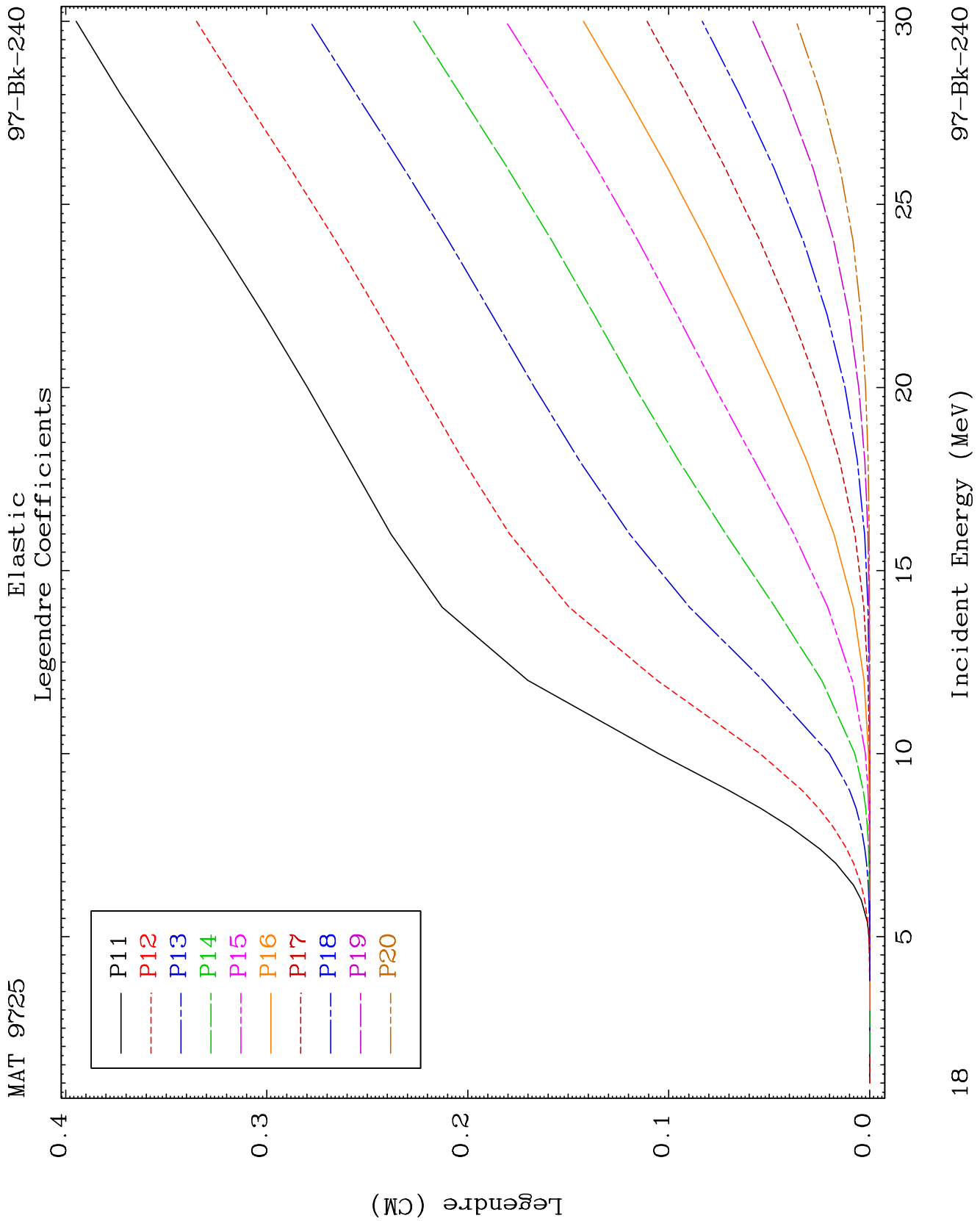


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Incident Energy (MeV)

97-Bk-240

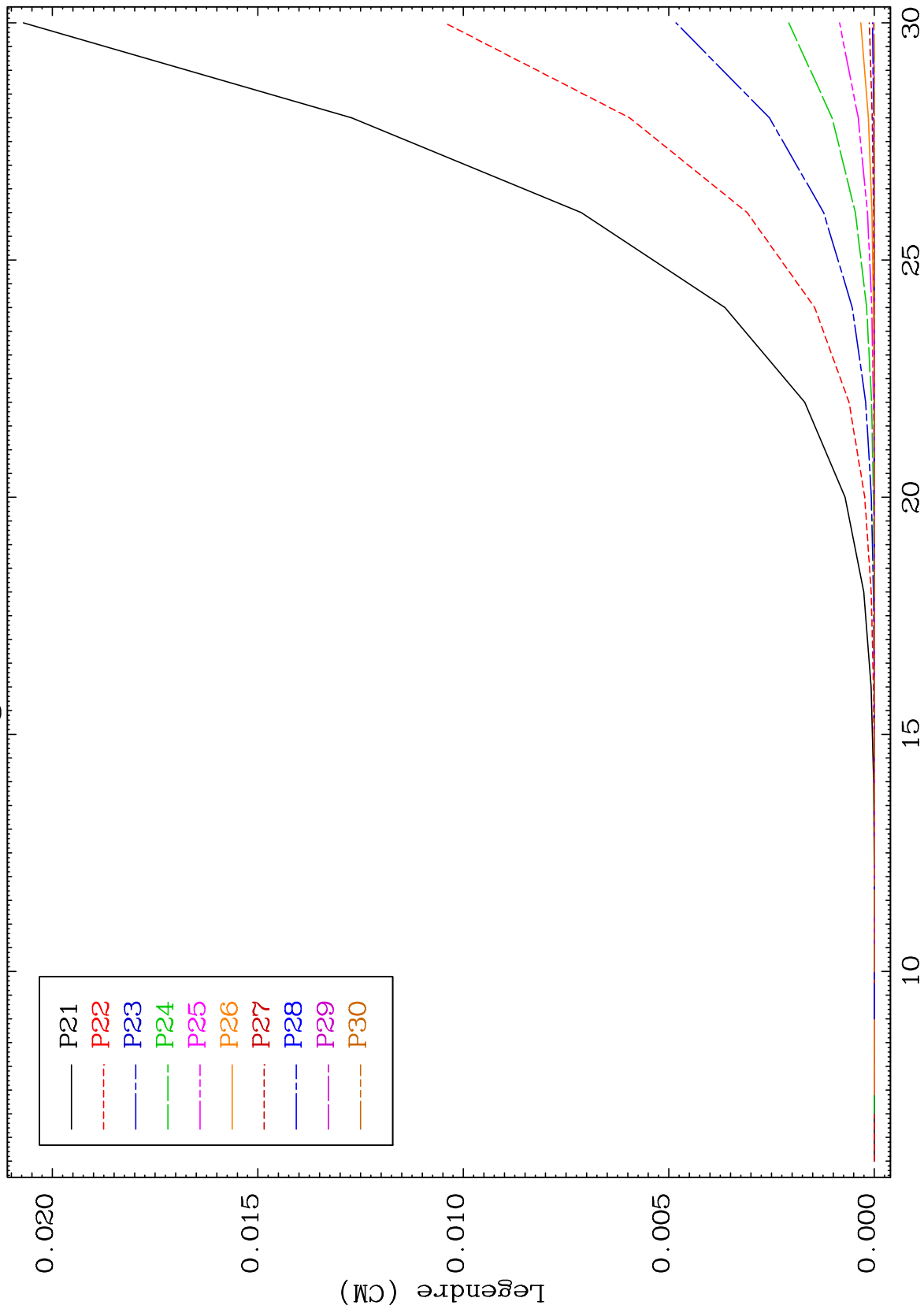




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Elastic Legendre Coefficients

97-Bk-240



19

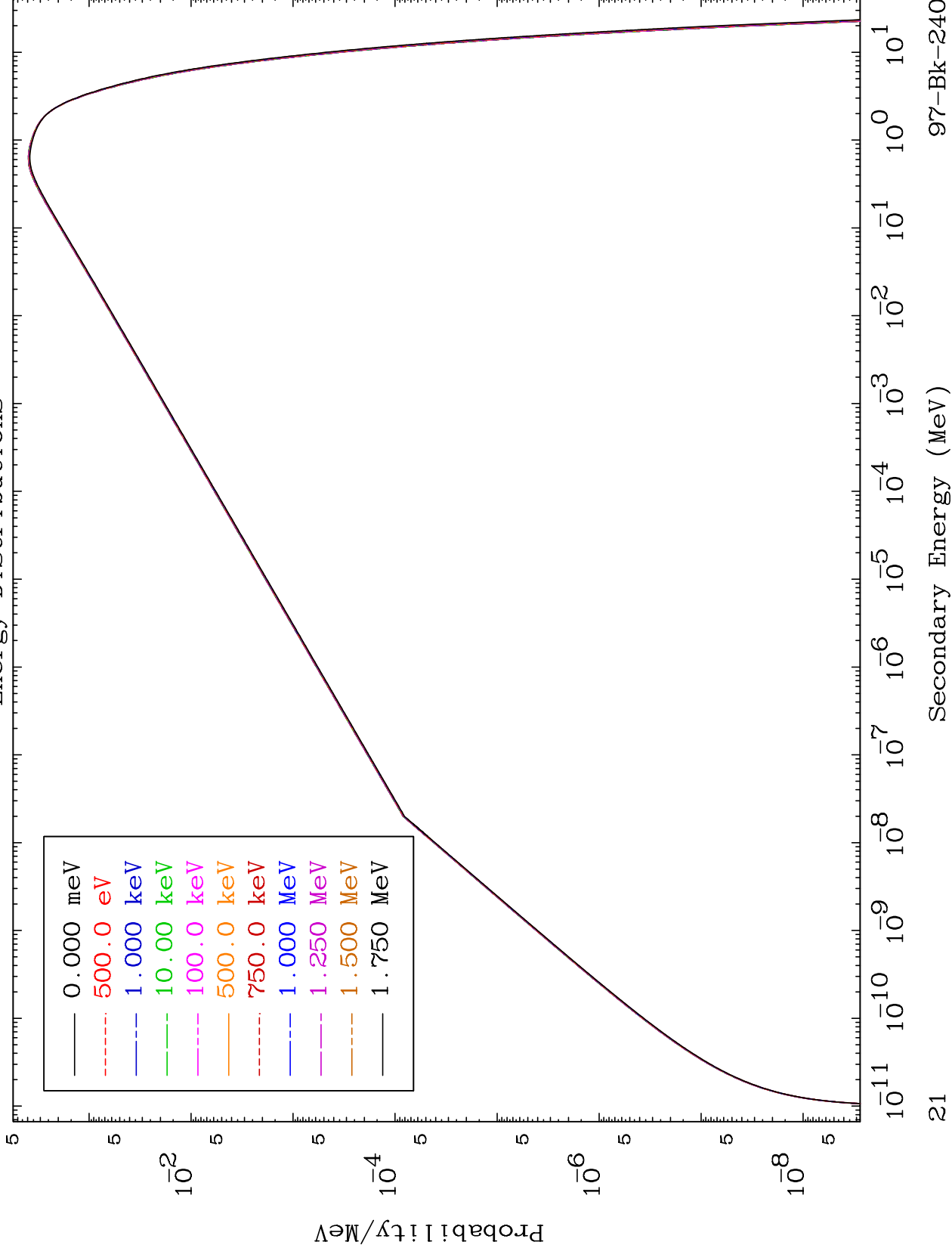
Incident Energy (MeV)

97-Bk-240

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Fission Energy Distributions

97-Bk-240



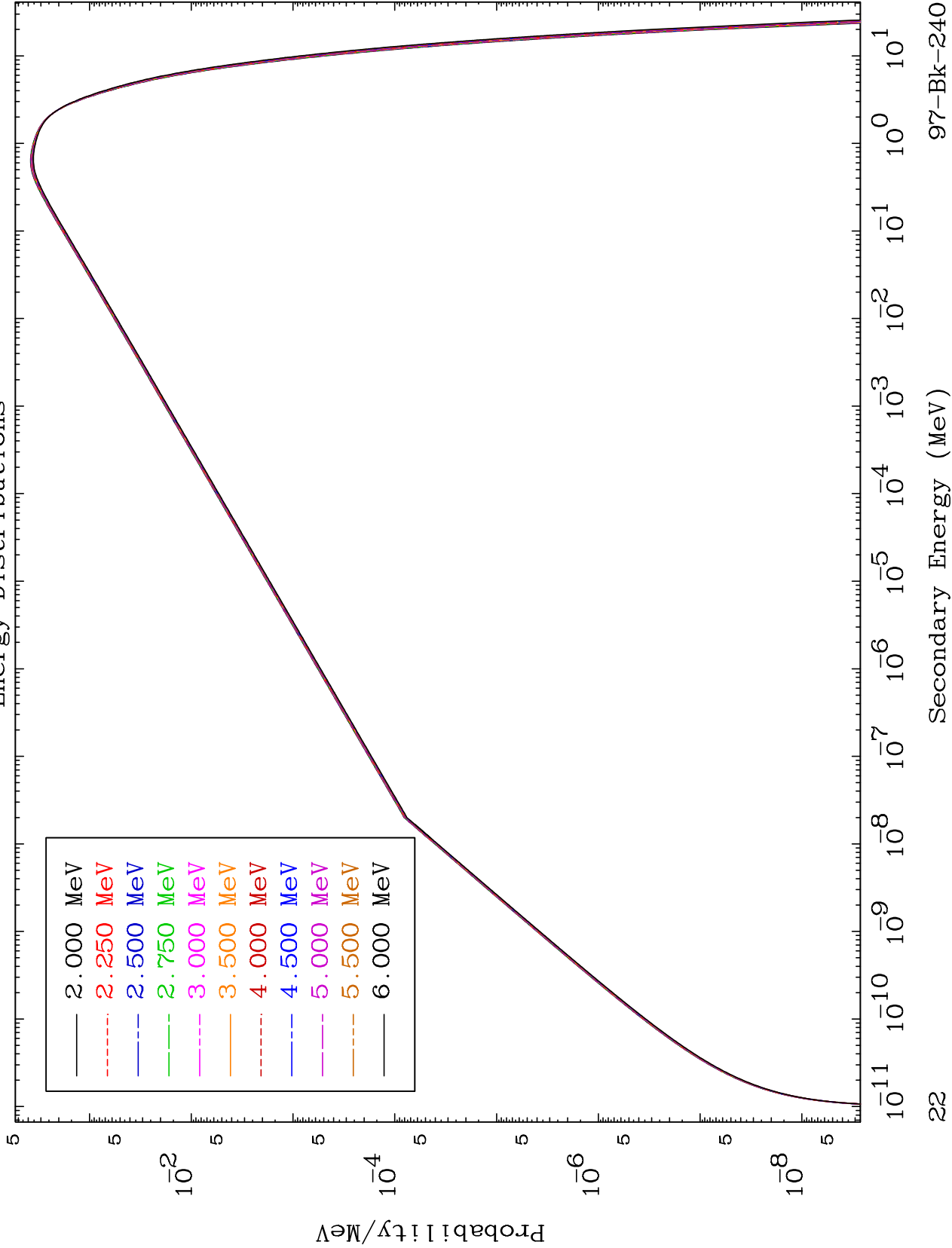
97-Bk-240

Secondary Energy (MeV)

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Fission Energy Distributions

97-Bk-240



97-Bk-240

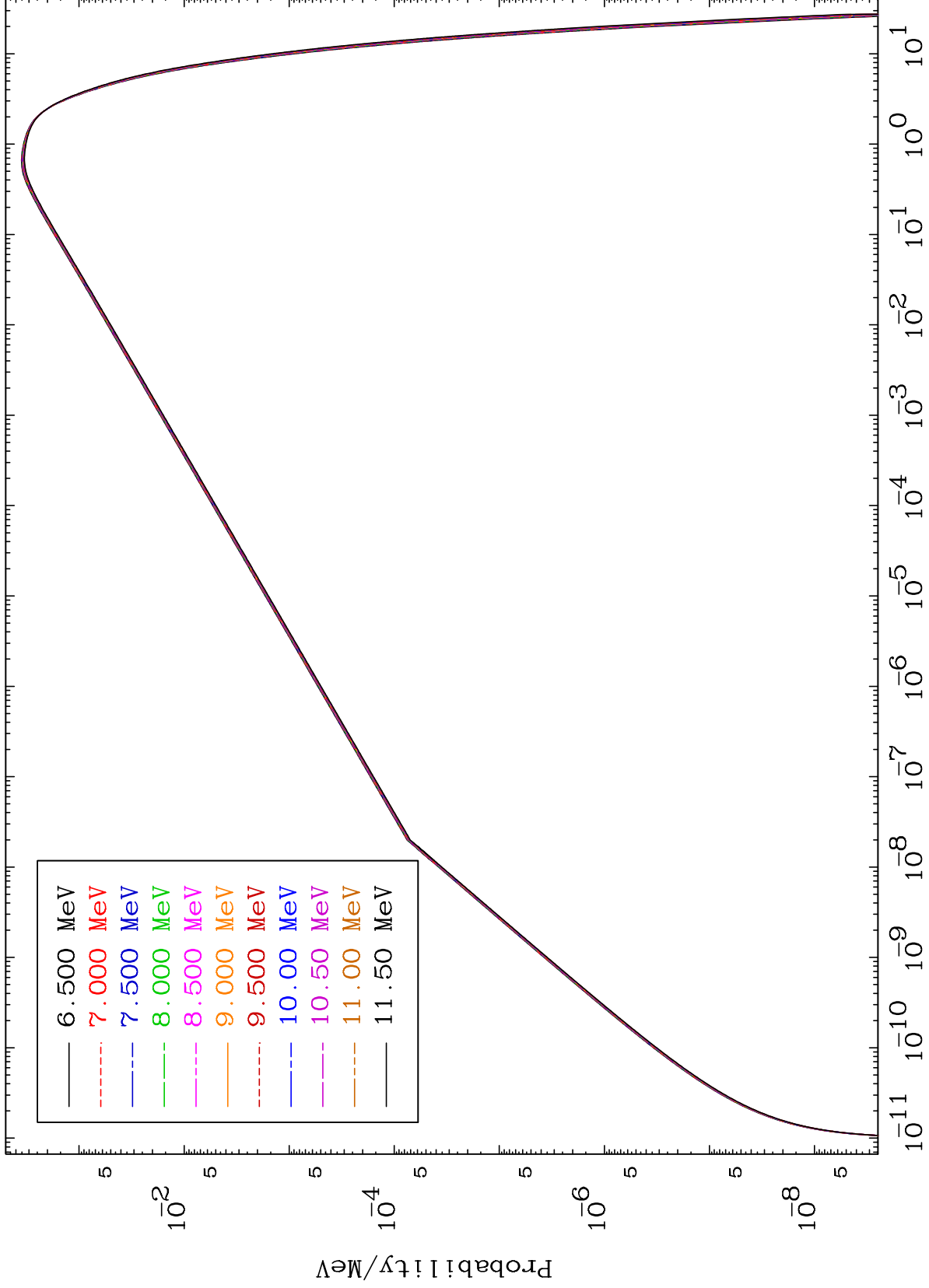
Secondary Energy (MeV)

22

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Fission Energy Distributions

97-Bk-240



23

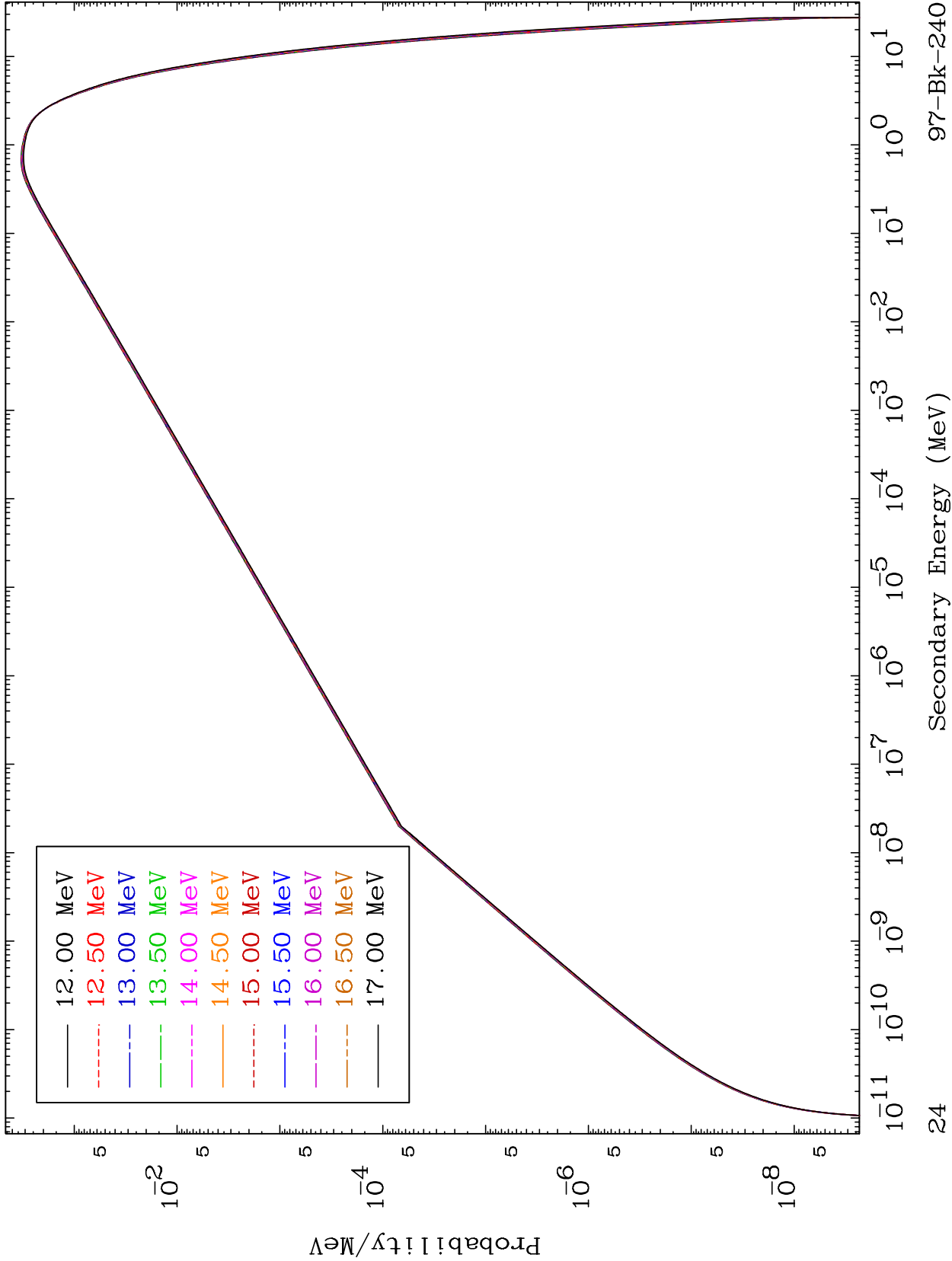
Secondary Energy (MeV)

97-Bk-240

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Fission Energy Distributions

97-Bk-240



97-Bk-240

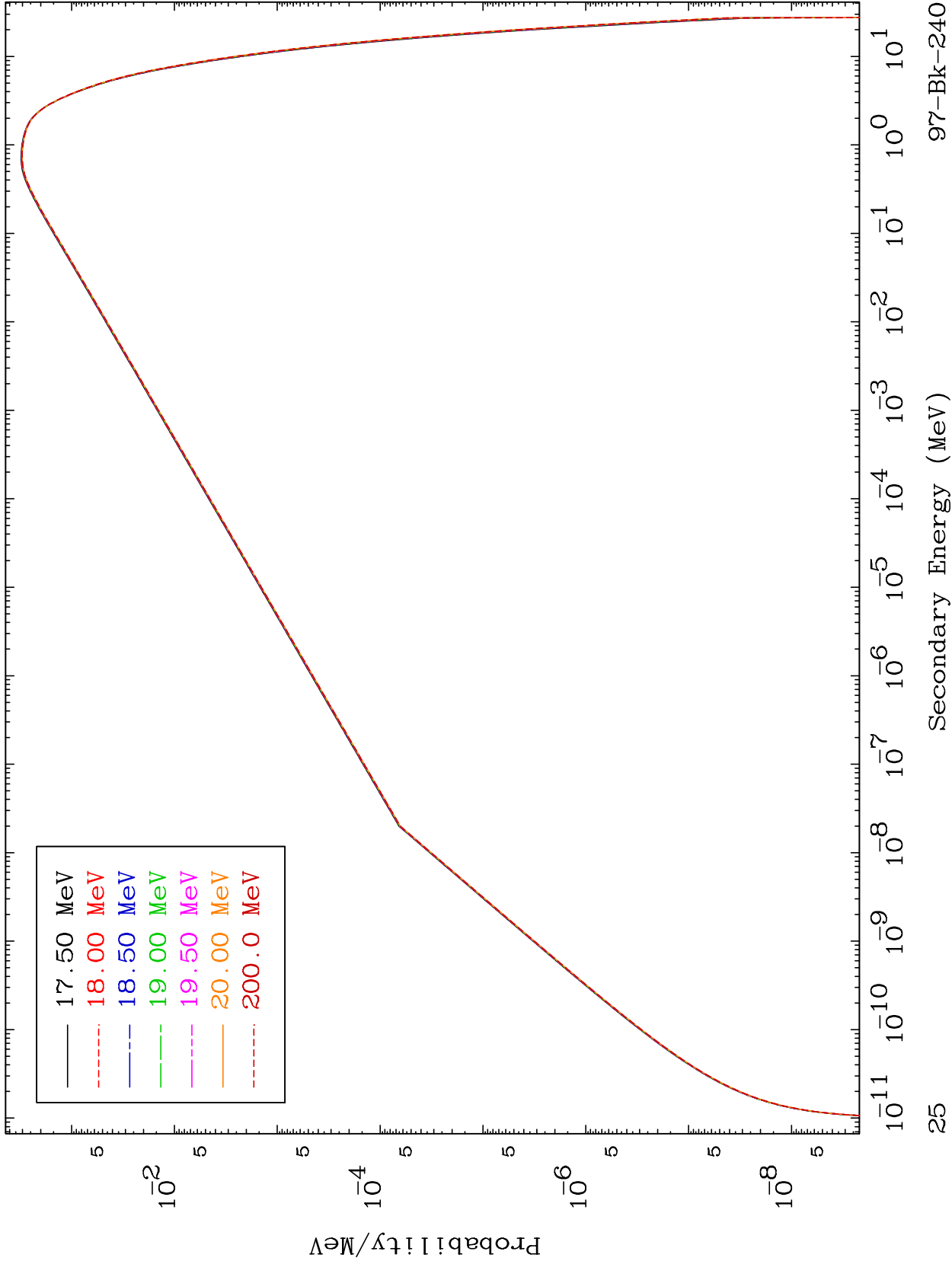
Secondary Energy (MeV)

24

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Fission Energy Distributions

97-Bk-240



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(n,n') α

97-Bk-240

Radionuclide Production Cross Section

