

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

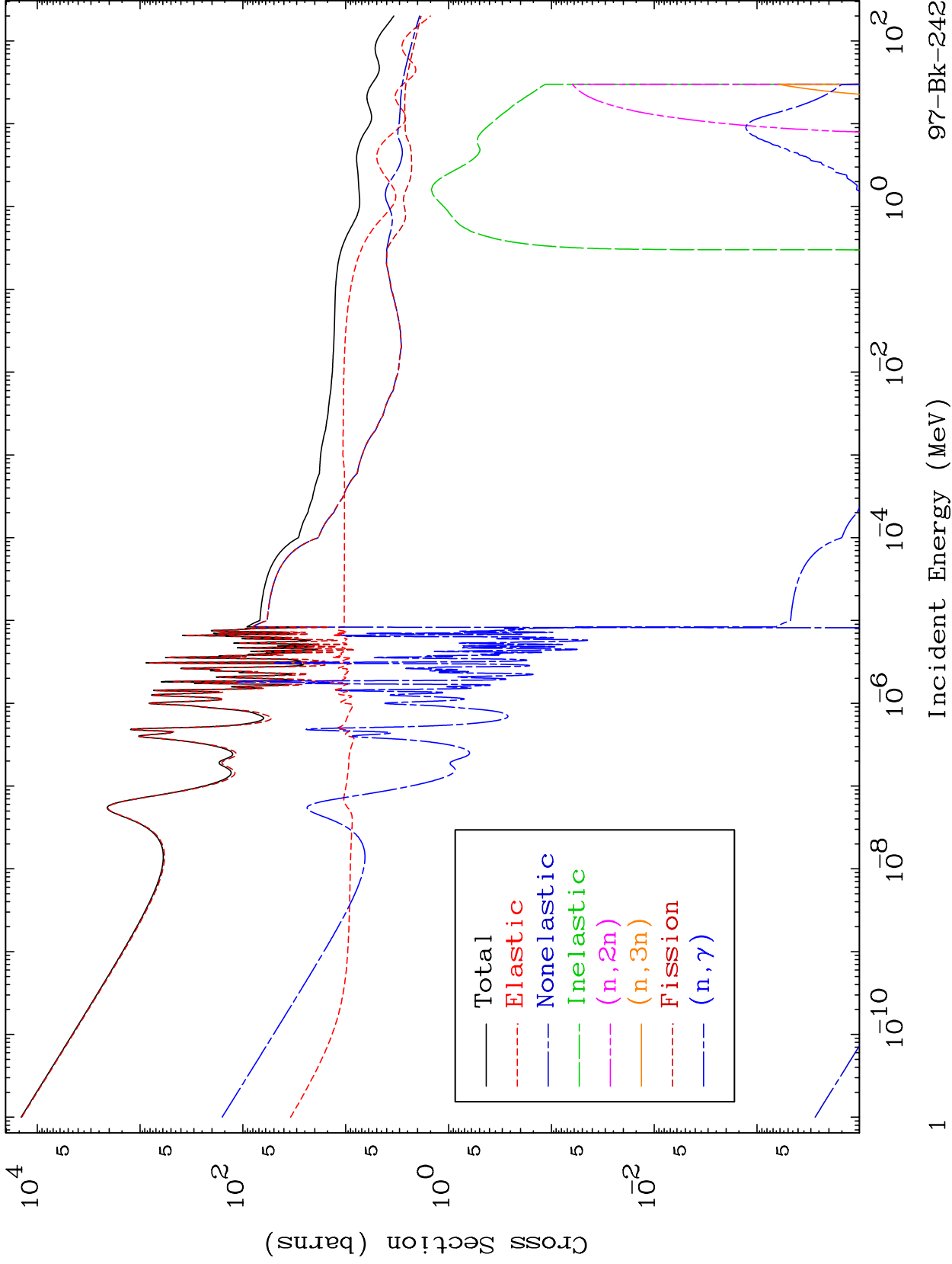
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Press Mouse Button to Start

MAT 9731

Neutron Major  
293 Kelvin Cross Sections

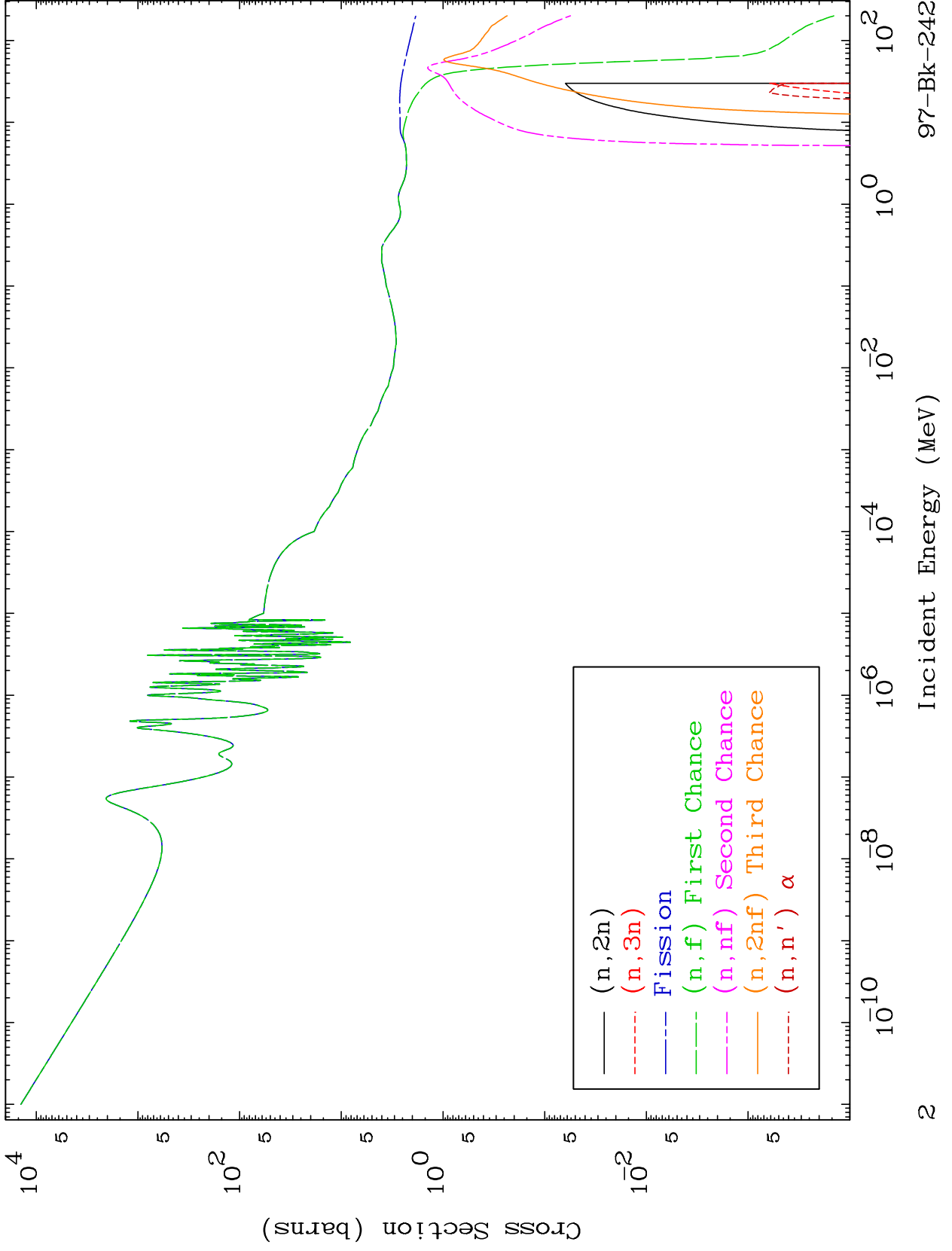
97-Bk-242



MAT 9731

Neutron Absorption  
293 Kelvin Cross Sections

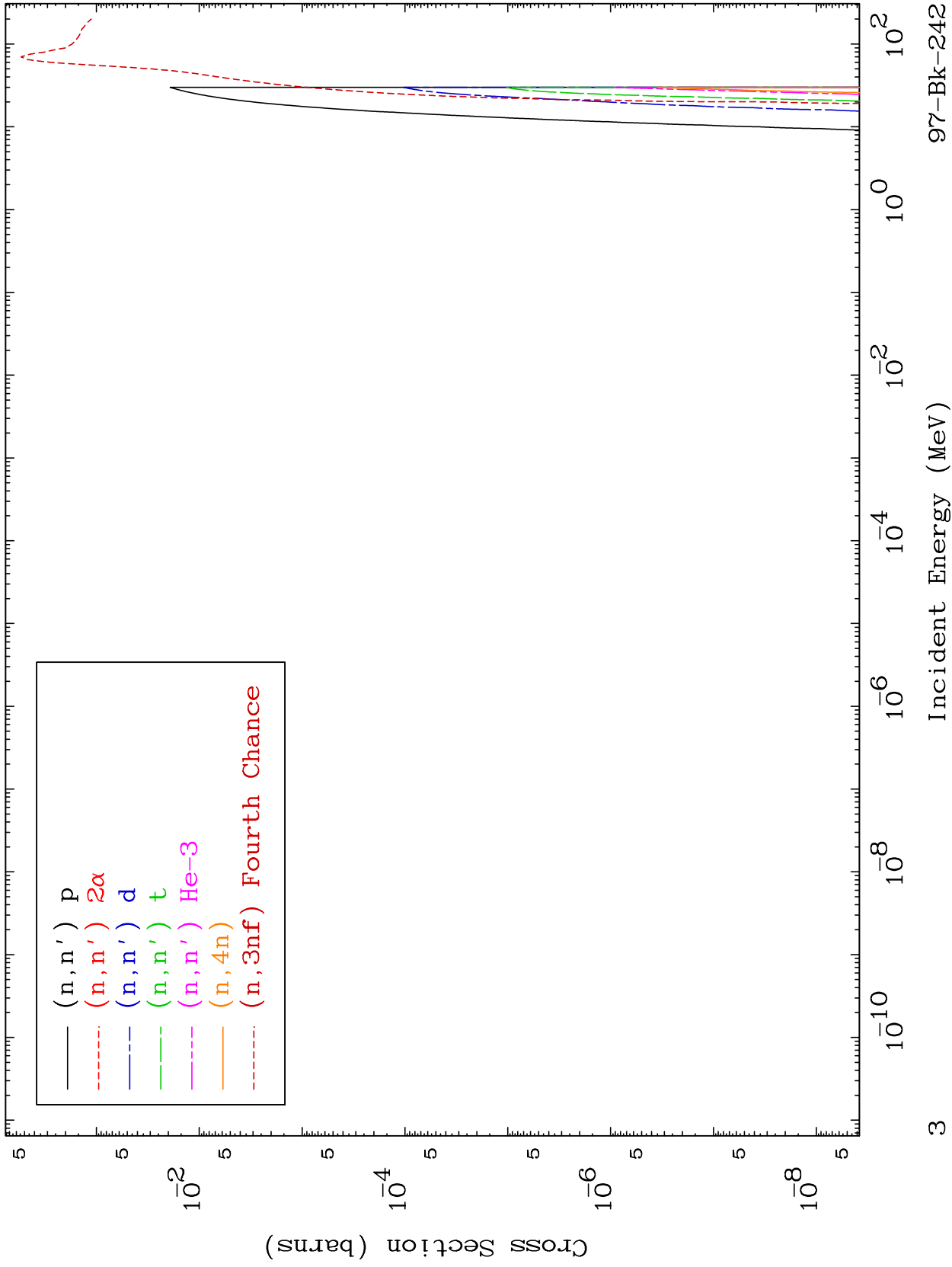
97-Bk-242



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Neutron Absorption  
293 Kelvin Cross Sections

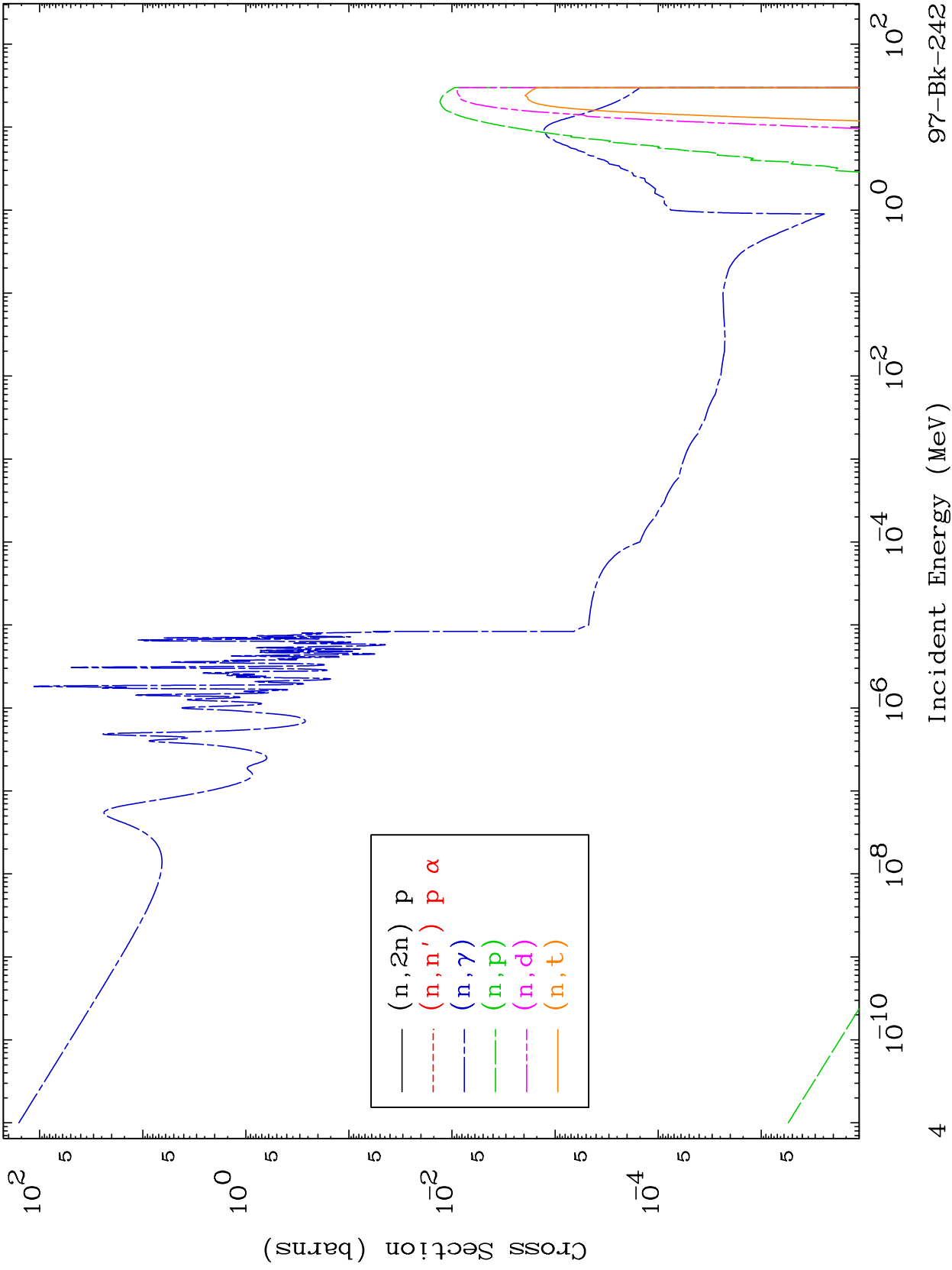
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Neutron Absorption  
293 Kelvin Cross Sections

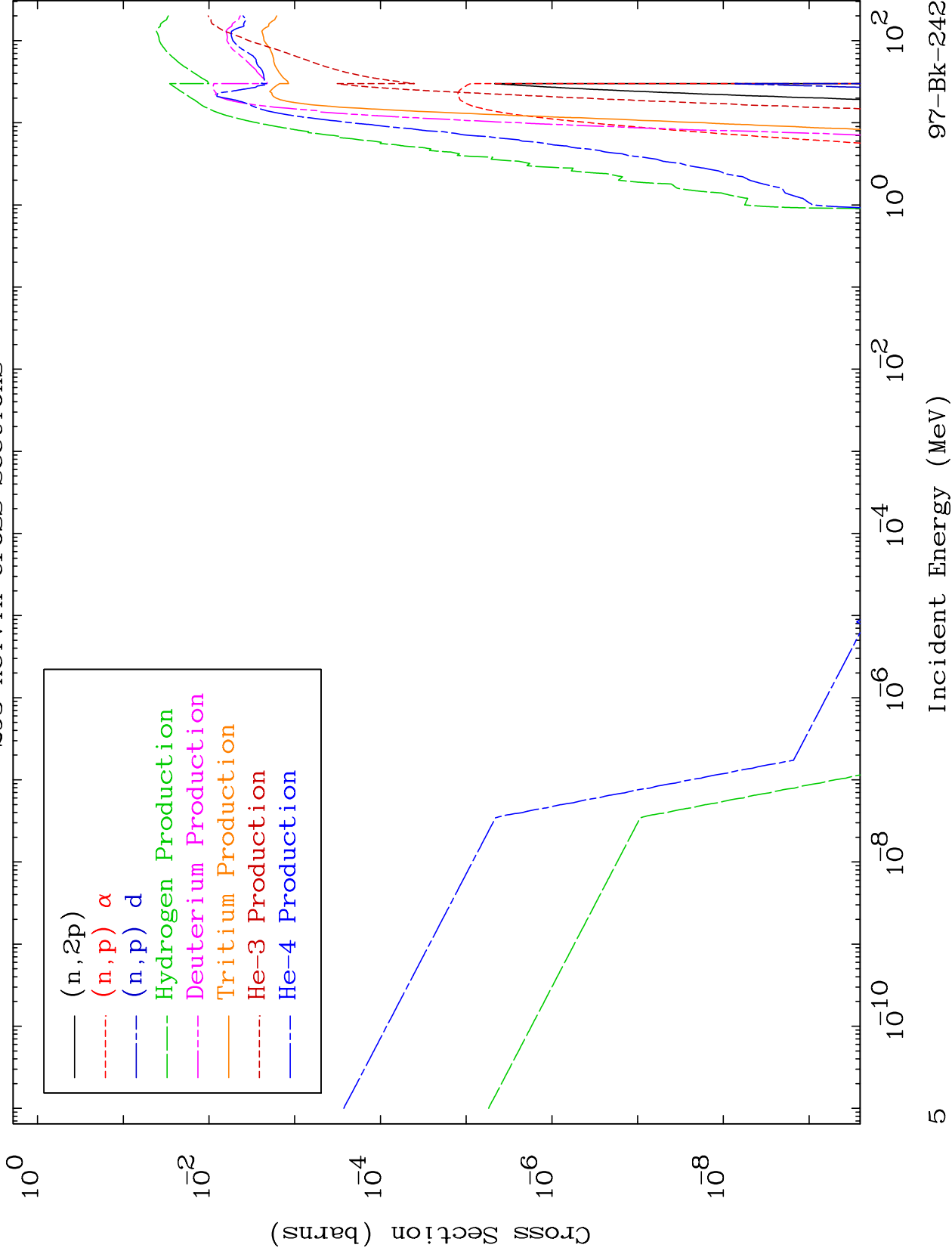
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Neutron Absorption  
293 Kelvin Cross Sections

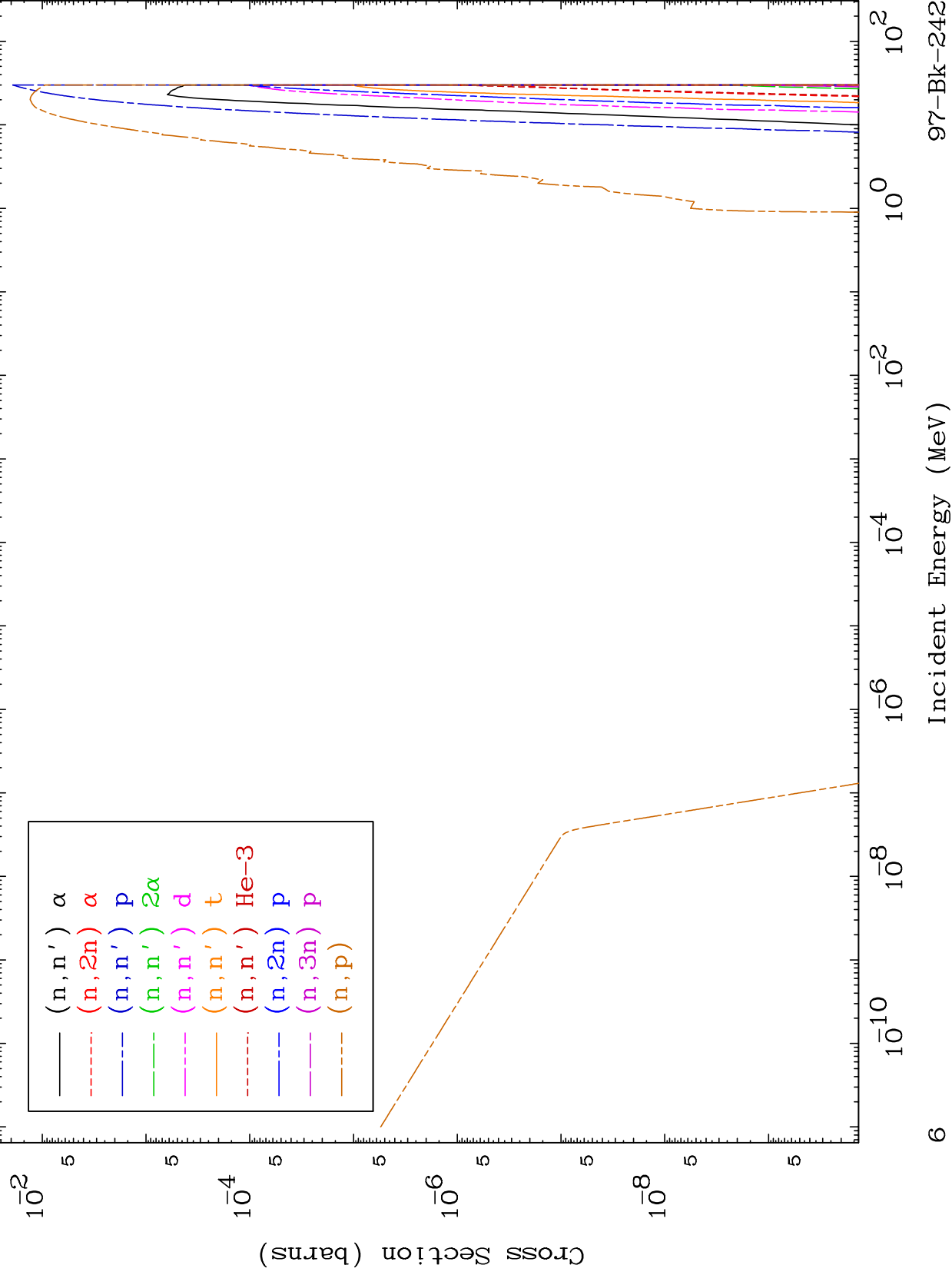
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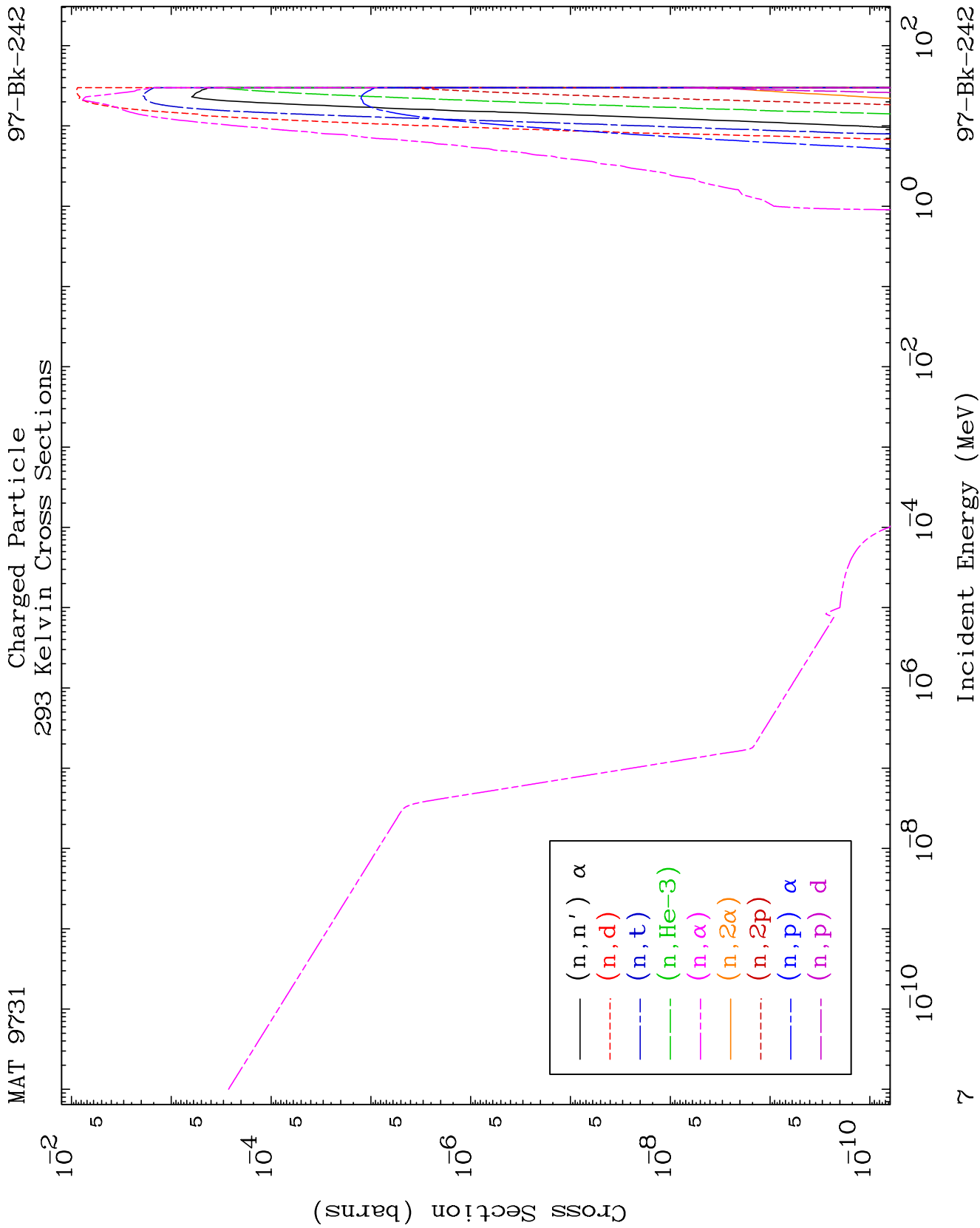


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Charged Particle  
293 Kelvin Cross Sections

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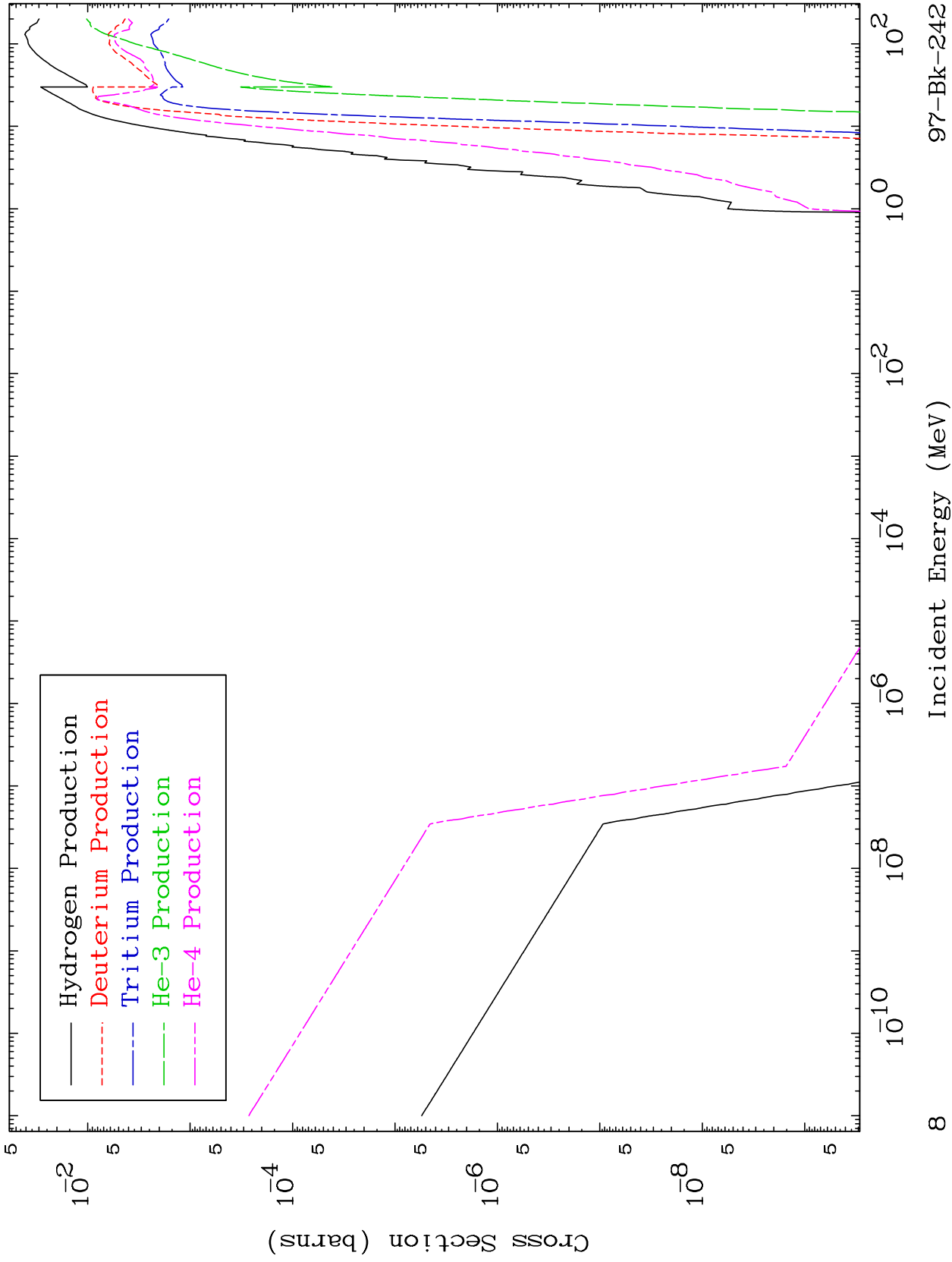




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Particle Production  
293 Kelvin Cross Sections

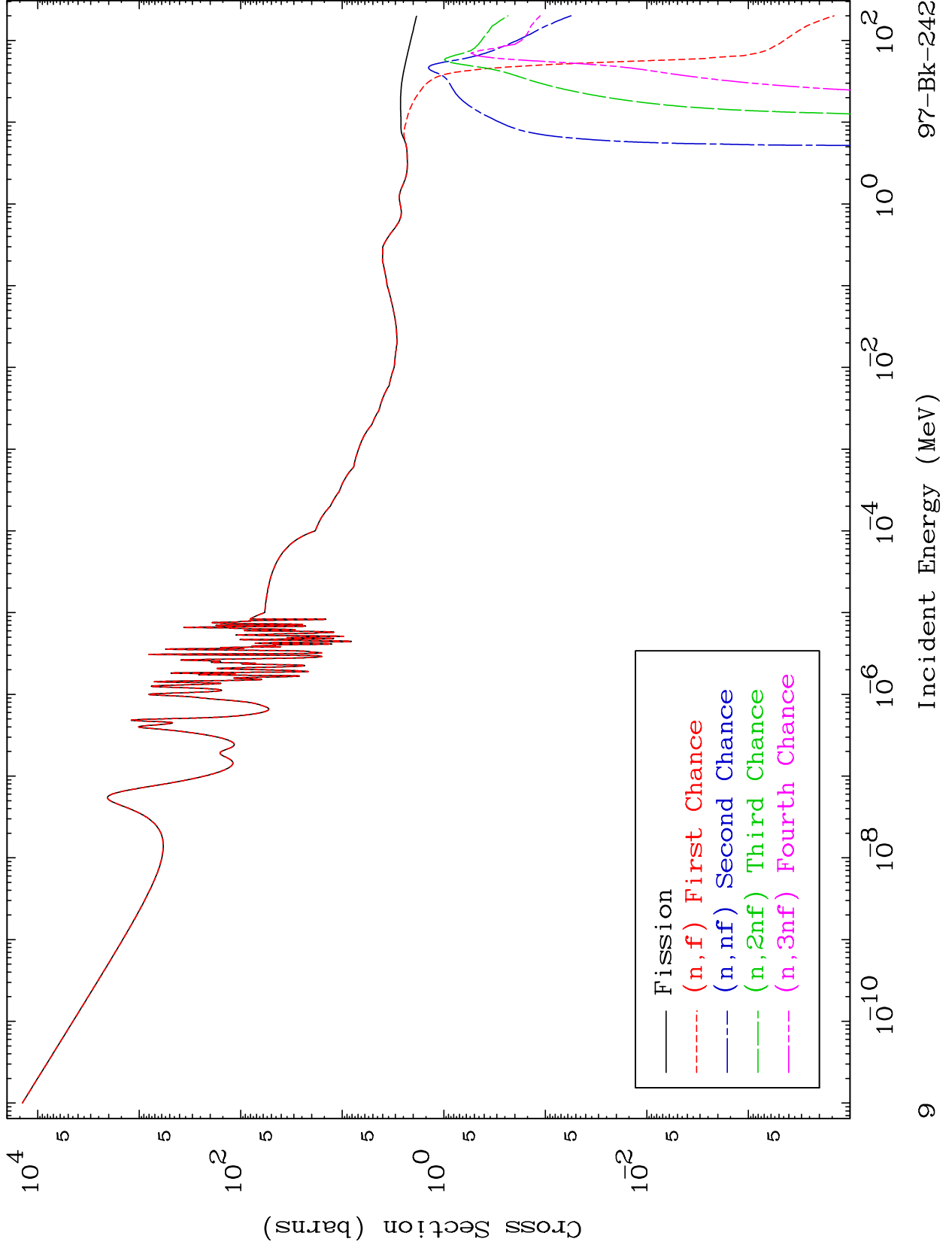
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Fission  
293 Kelvin Cross Sections

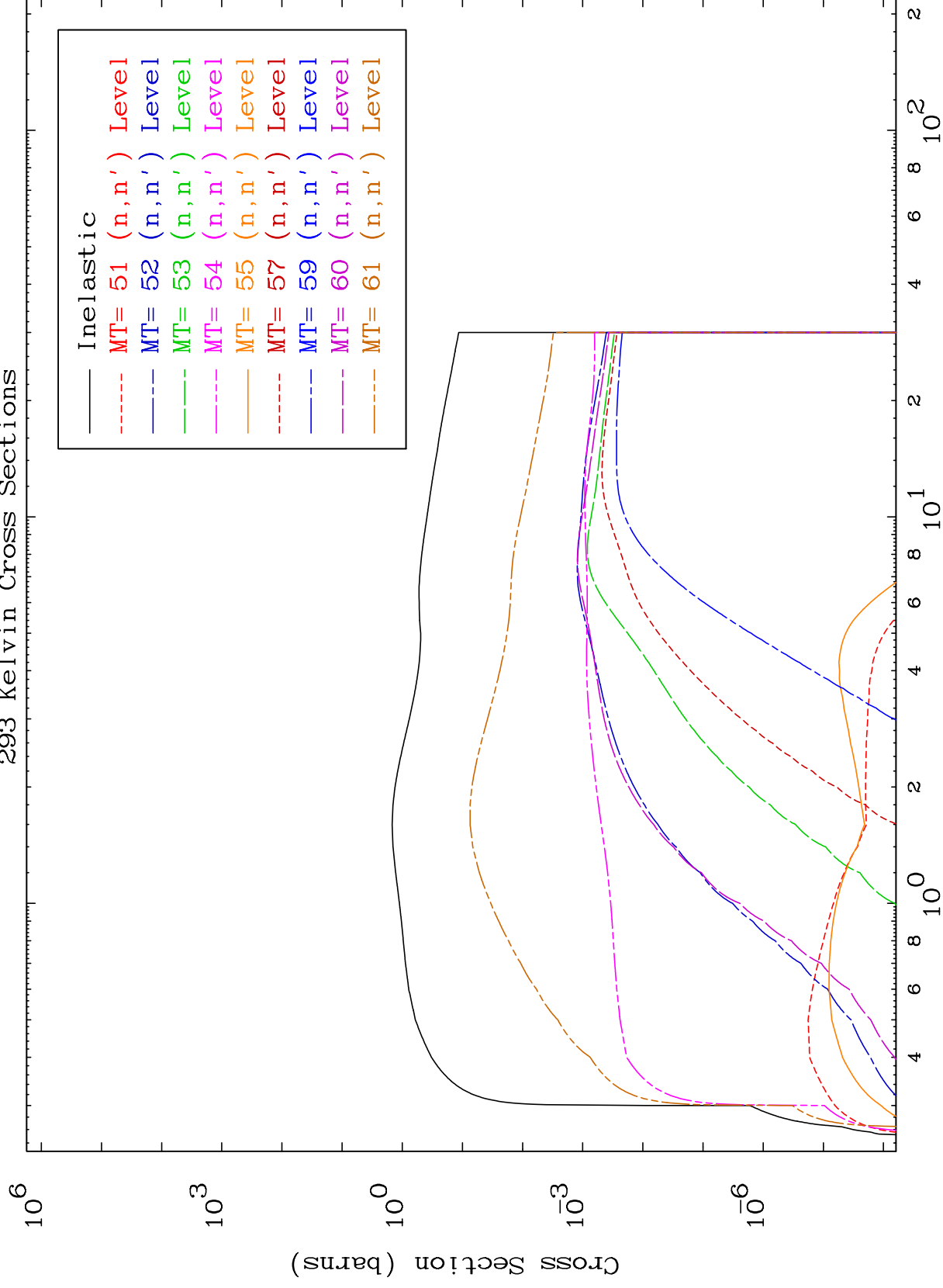
97-Bk-242



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(n,n') Levels  
293 Kelvin Cross Sections

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Incident Energy (MeV)

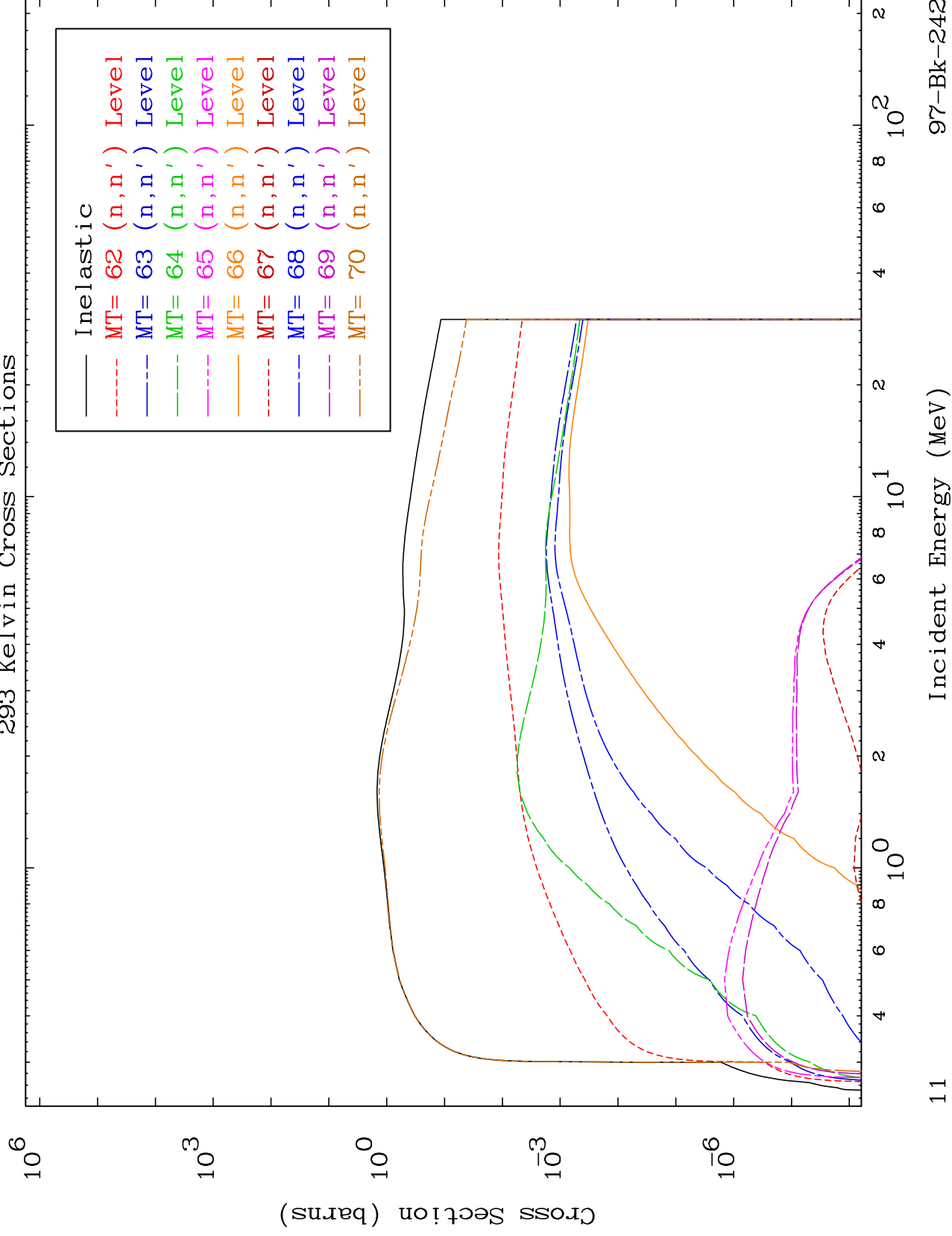
97-Bk-242

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(n,n') Levels

97-Bk-242

293 Kelvin Cross Sections

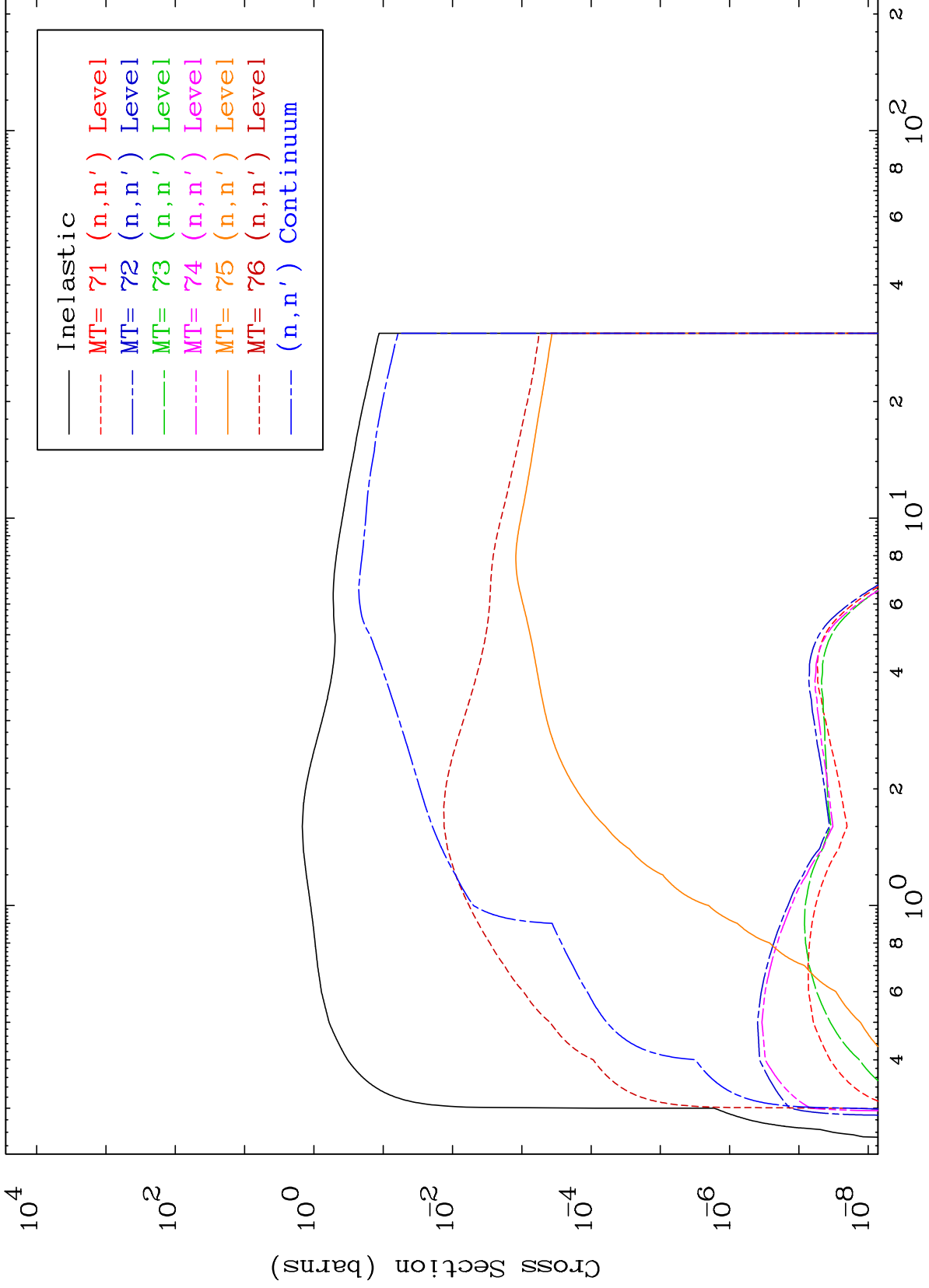


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(n,n') Levels

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293 Kelvin Cross Sections



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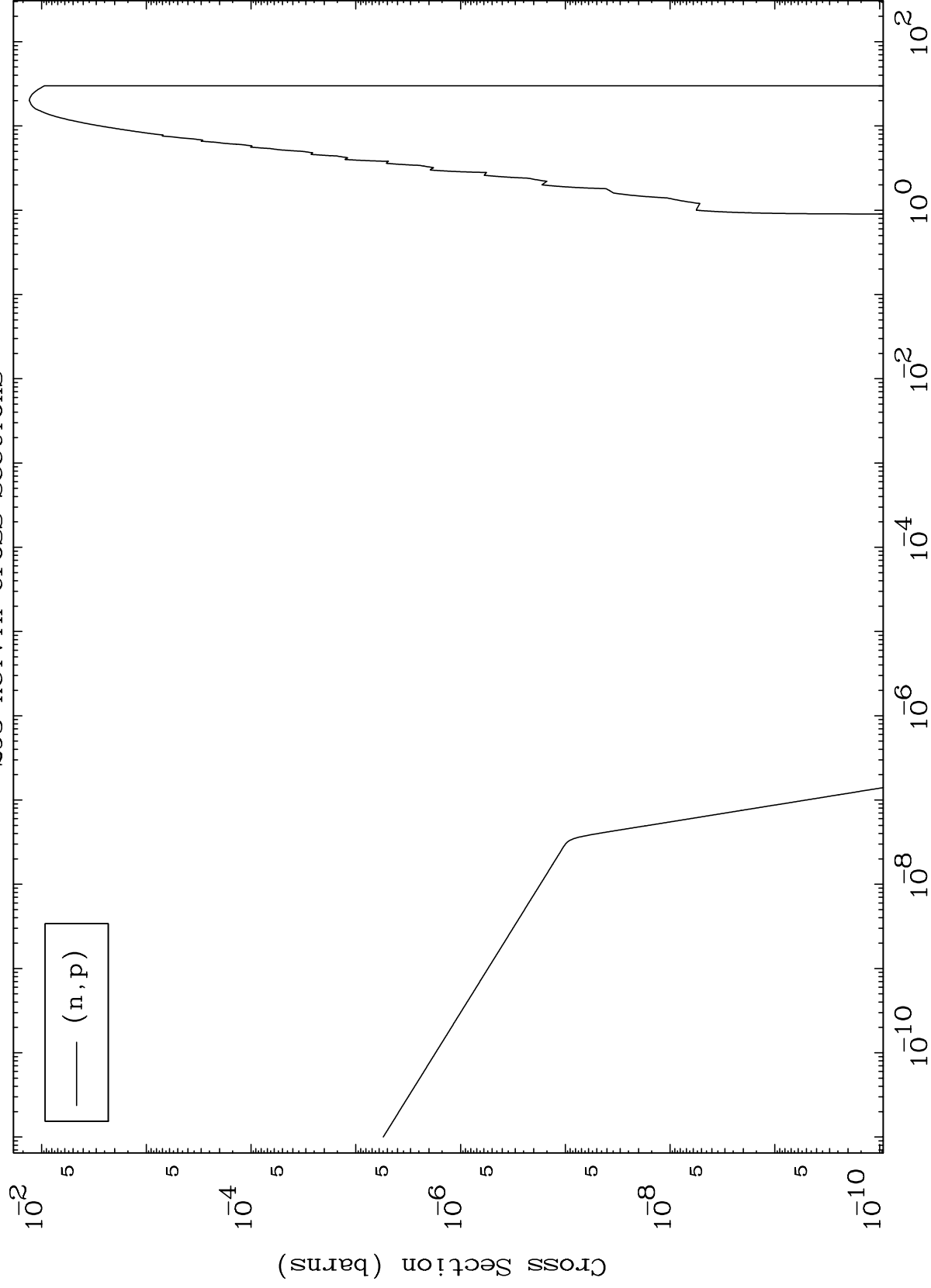
Incident Energy (MeV)

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(n,p) Levels  
293 Kelvin Cross Sections

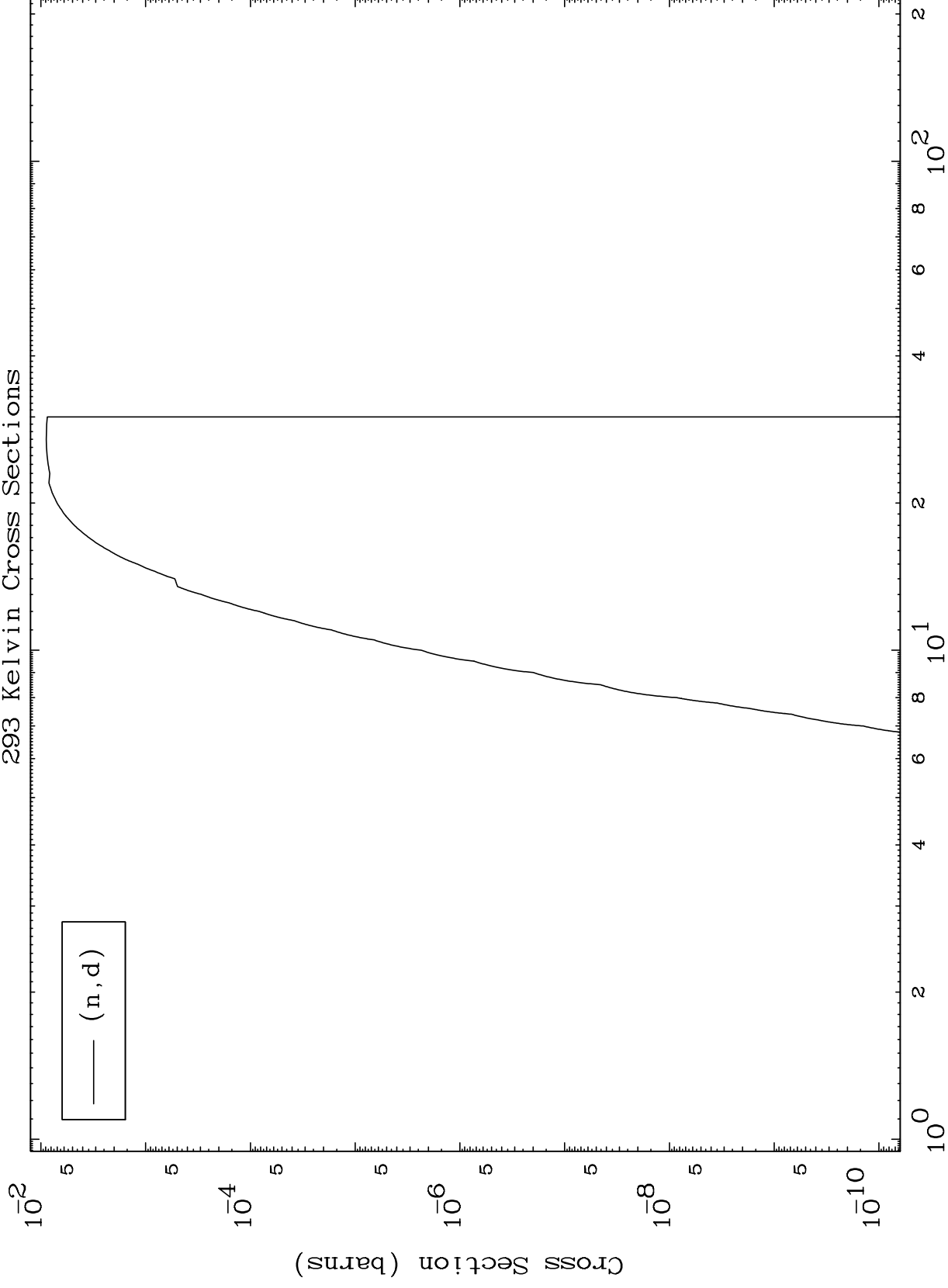
97-Bk-242



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(n,d) Levels  
293 Kelvin Cross Sections

97-Bk-242



14

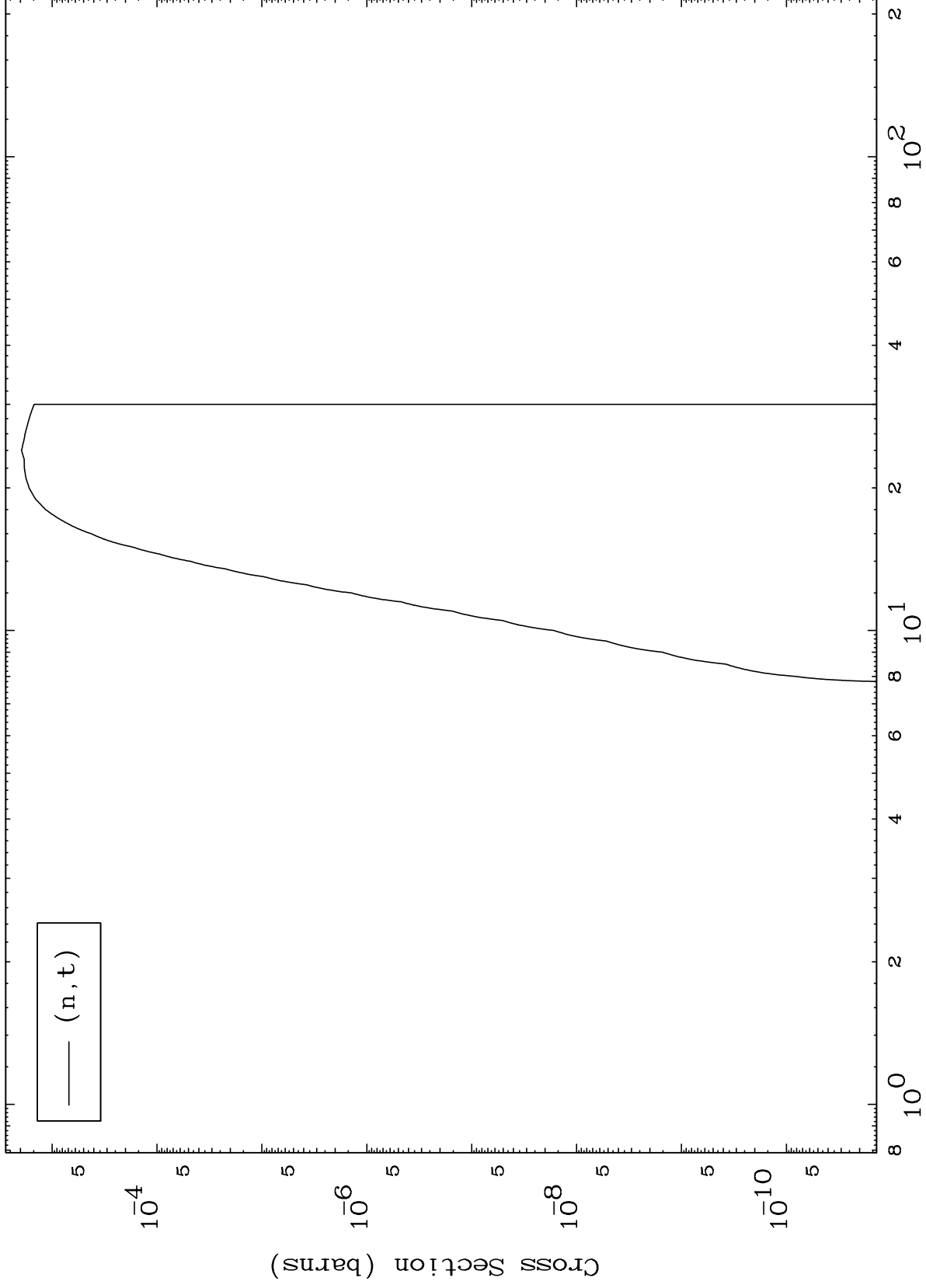
Incident Energy (MeV)

97-Bk-242

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(n,t) Levels  
293 Kelvin Cross Sections

97-Bk-242



15

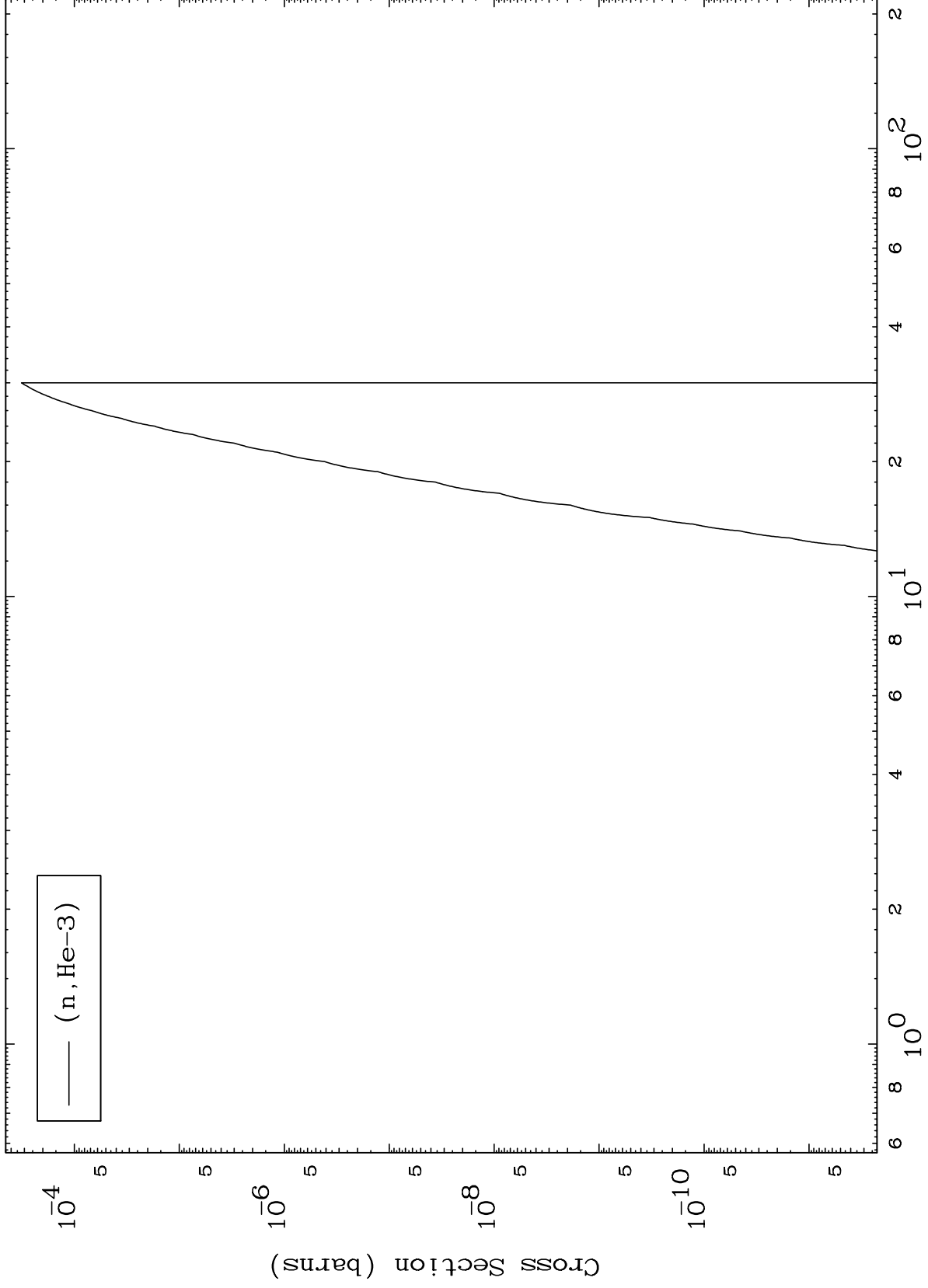
Incident Energy (MeV)

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(n,He3) Levels  
293 Kelvin Cross Sections

97-Bk-242



16

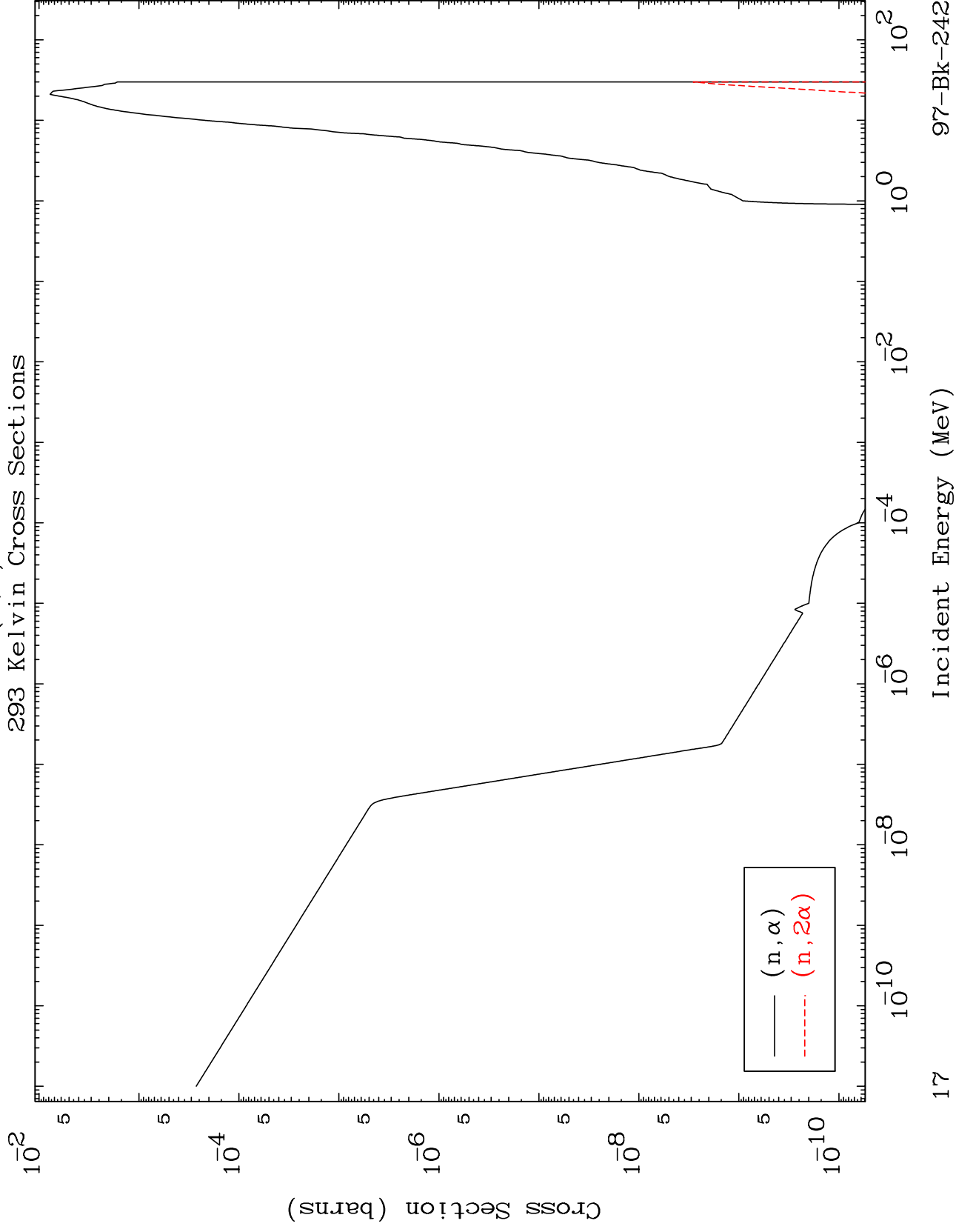
Incident Energy (MeV)

97-Bk-242

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(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

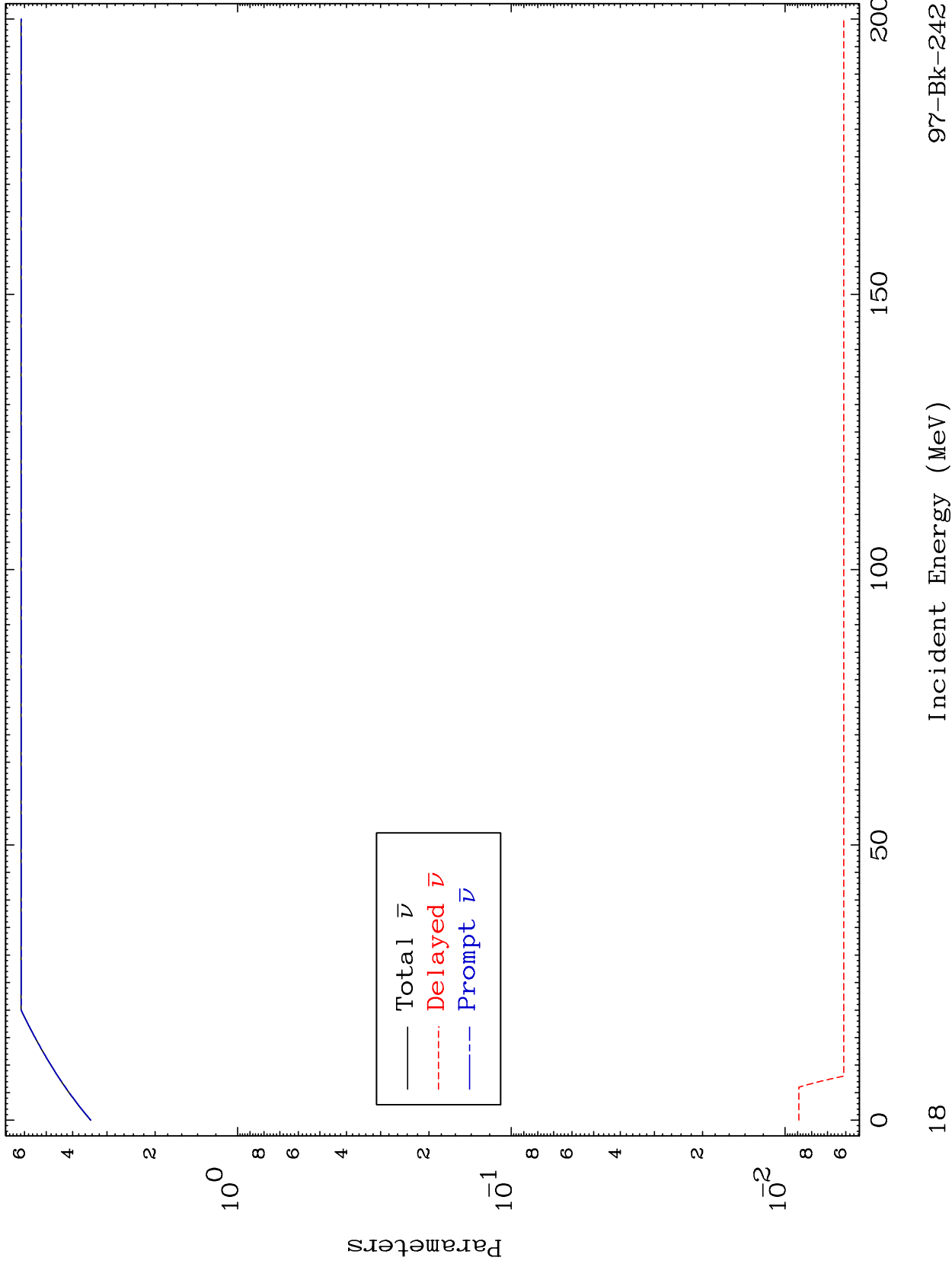
97-Bk-242



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Energy Release  
Parameters

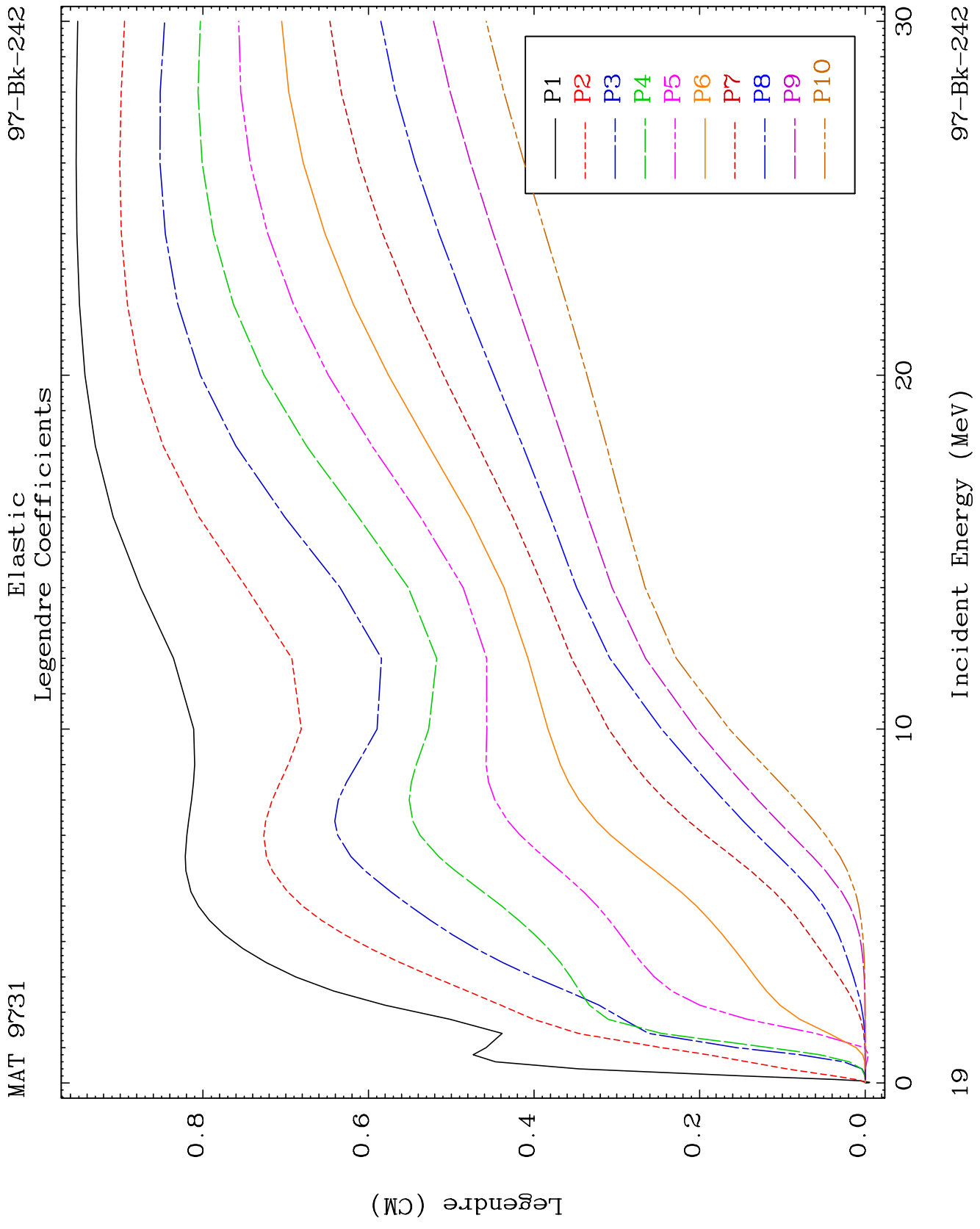
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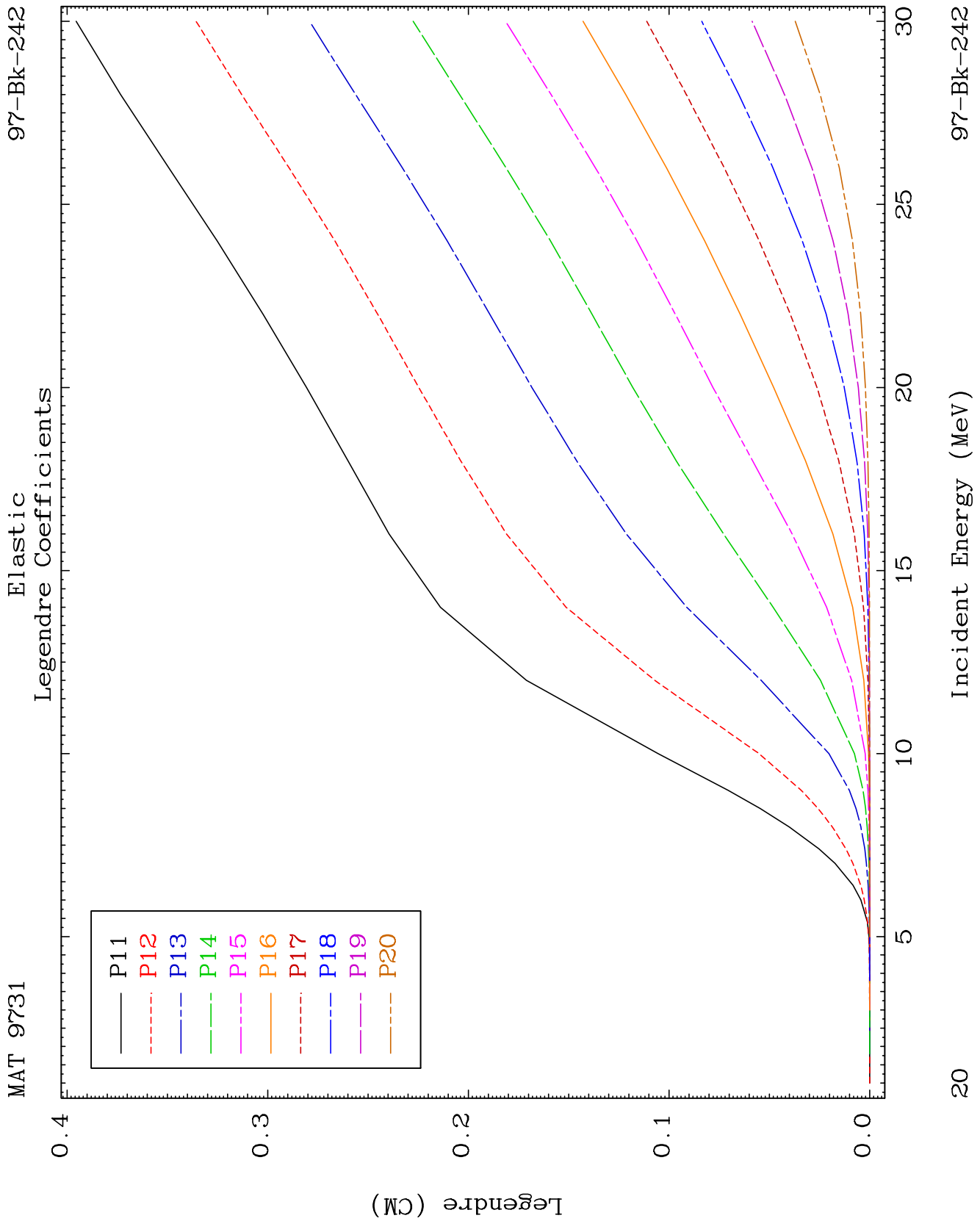


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Incident Energy (MeV)

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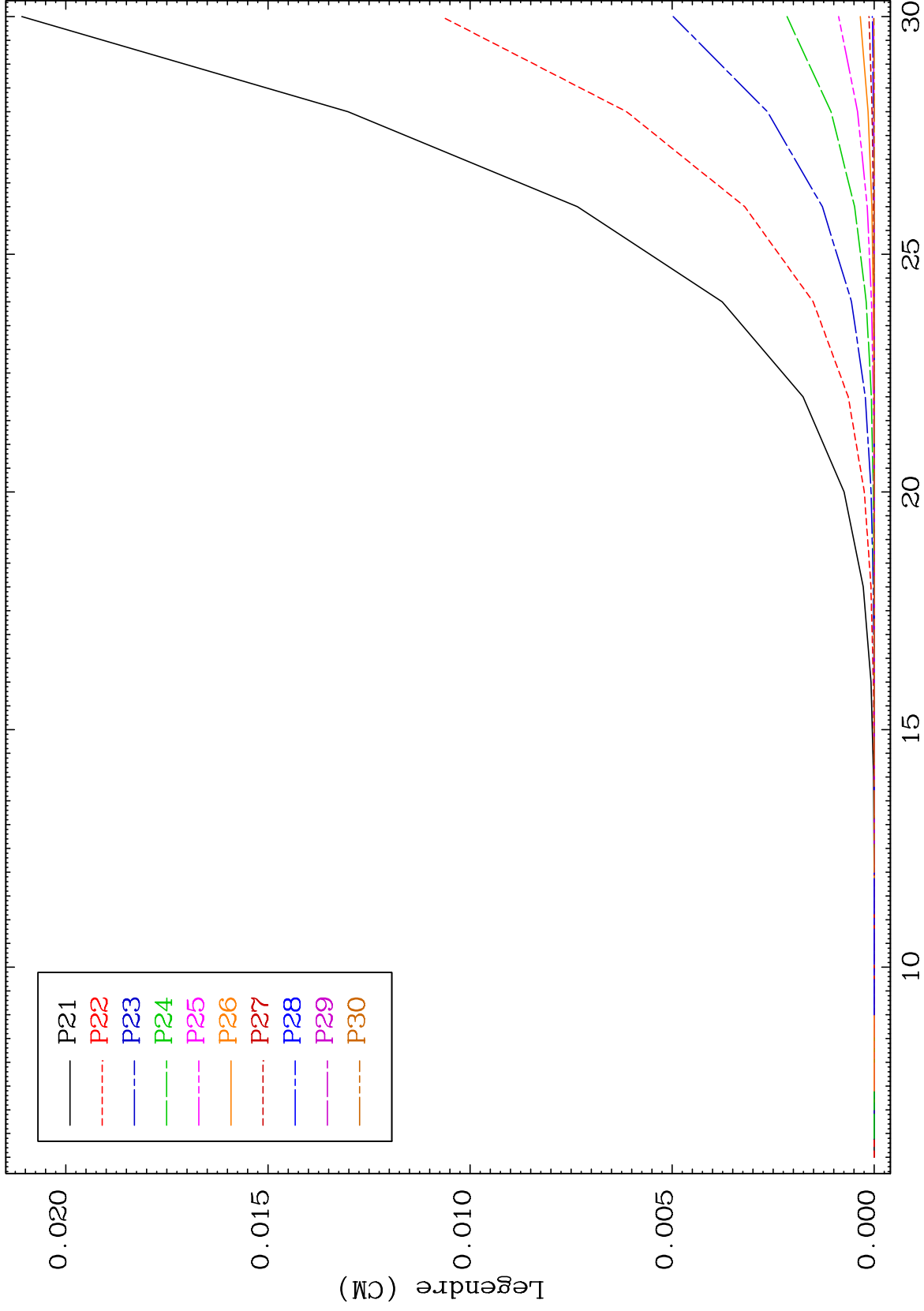




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### Elastic Legendre Coefficients

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Incident Energy (MeV)

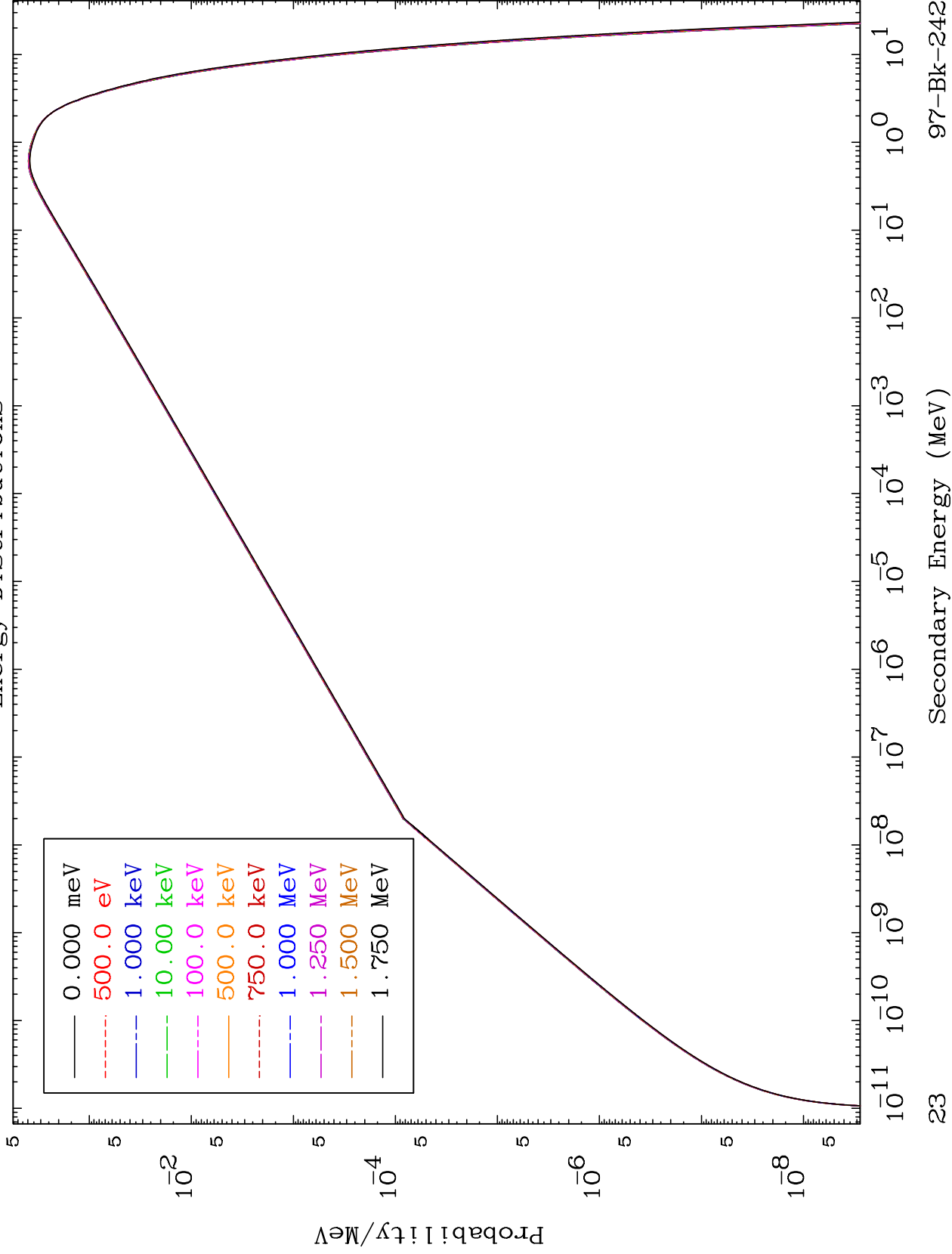
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### Fission Energy Distributions

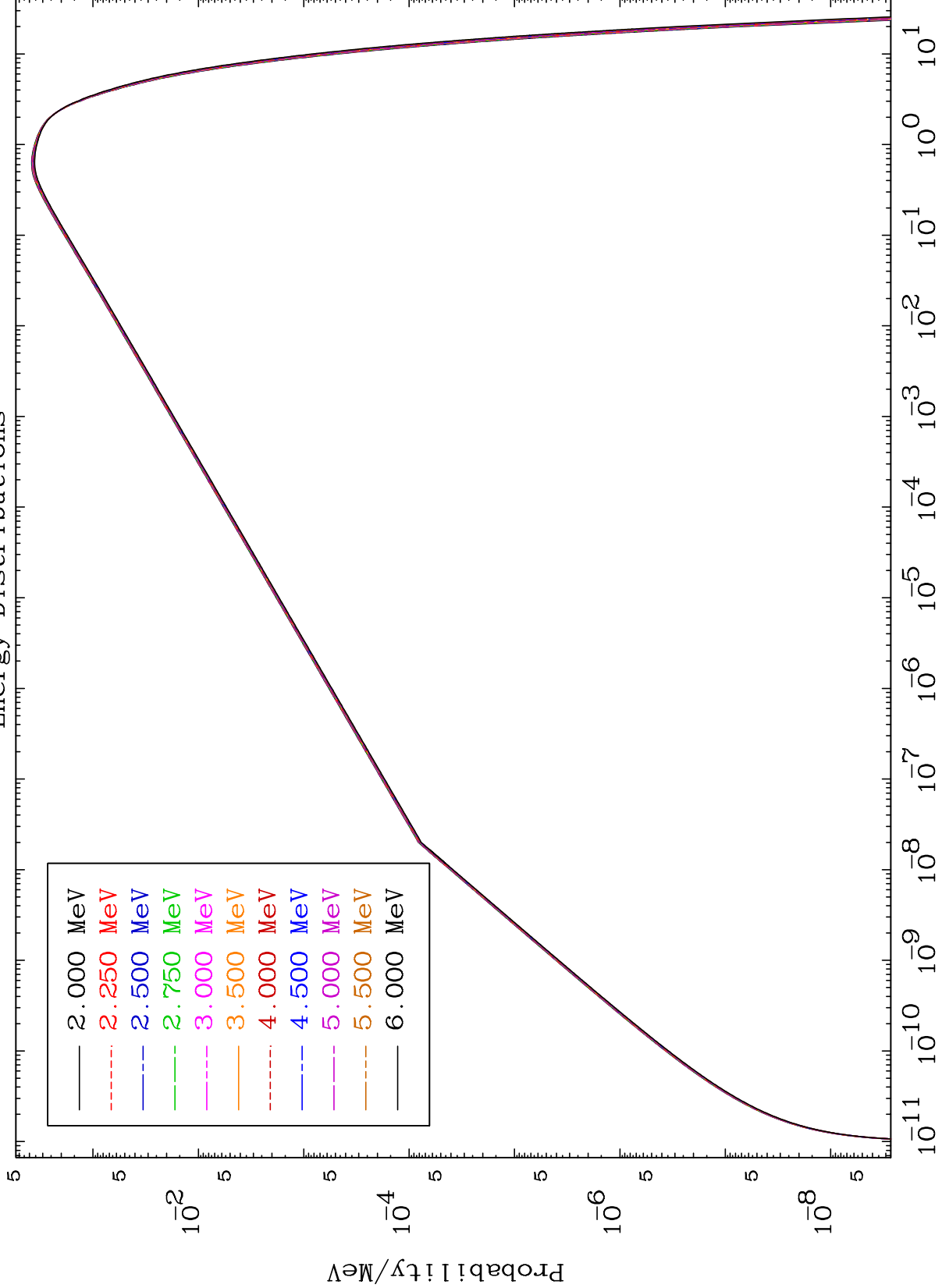
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### Fission Energy Distributions

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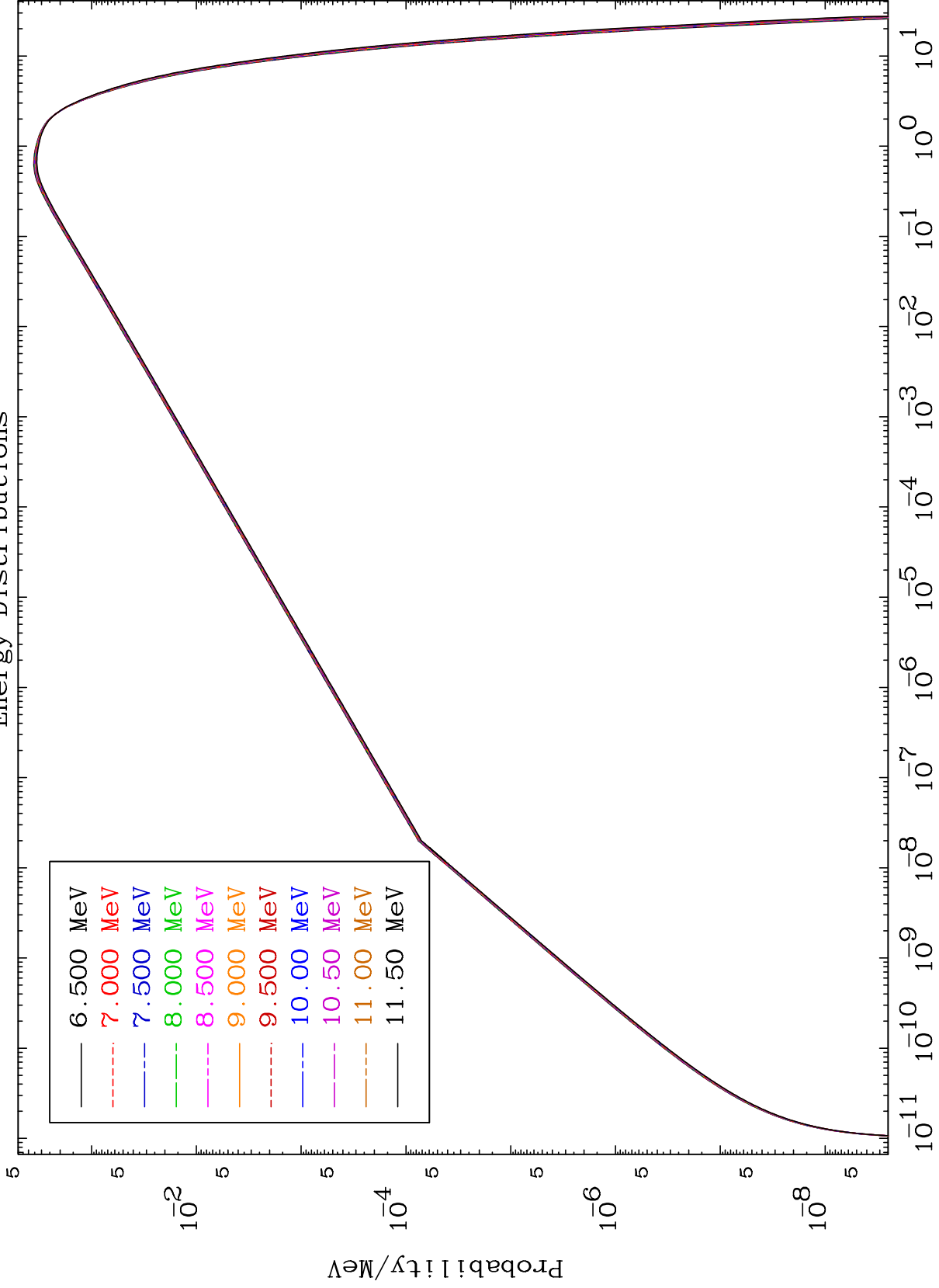
Secondary Energy (MeV)

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### Fission Energy Distributions

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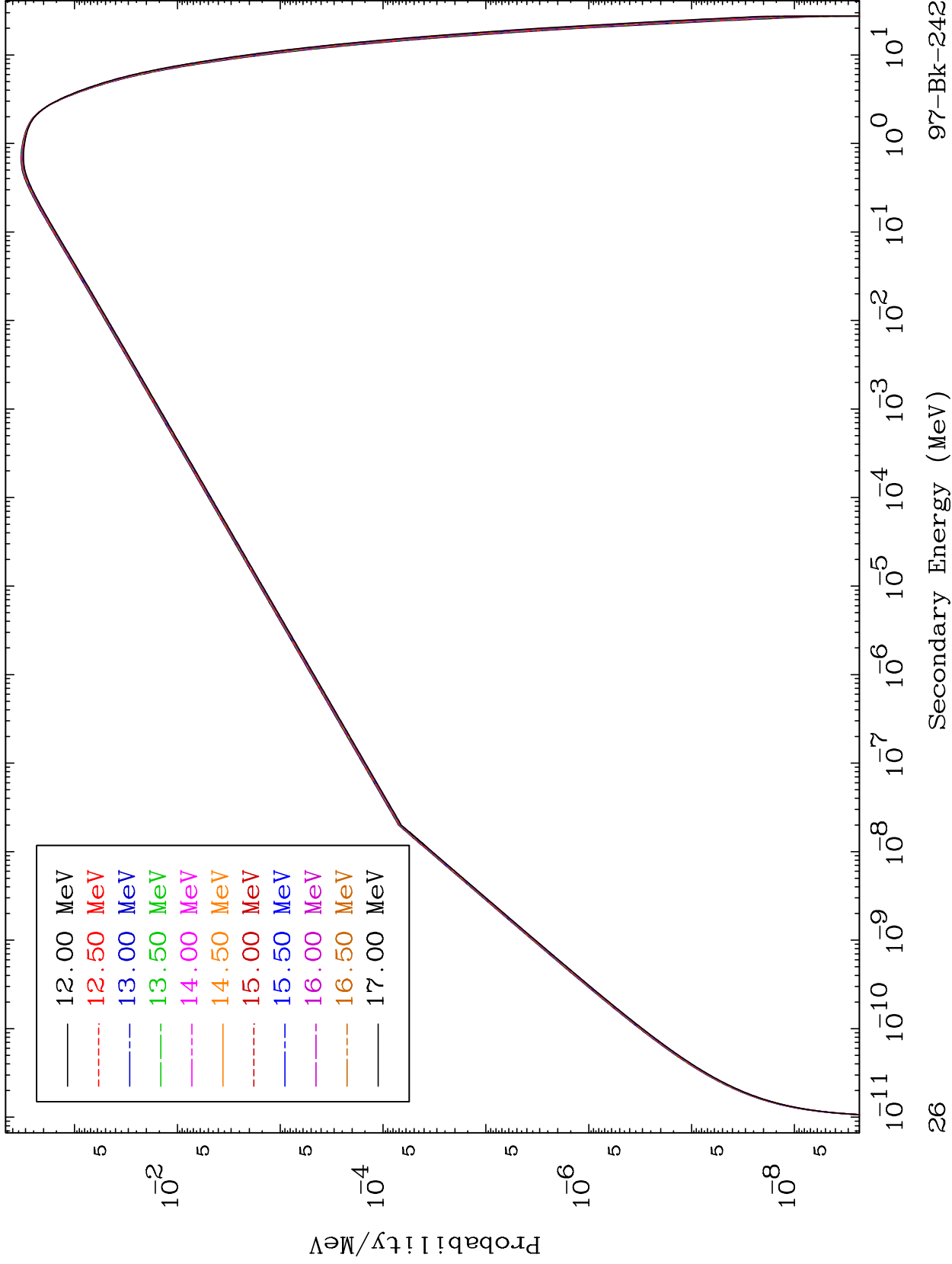
Secondary Energy (MeV)

97-Bk-242

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### Fission Energy Distributions

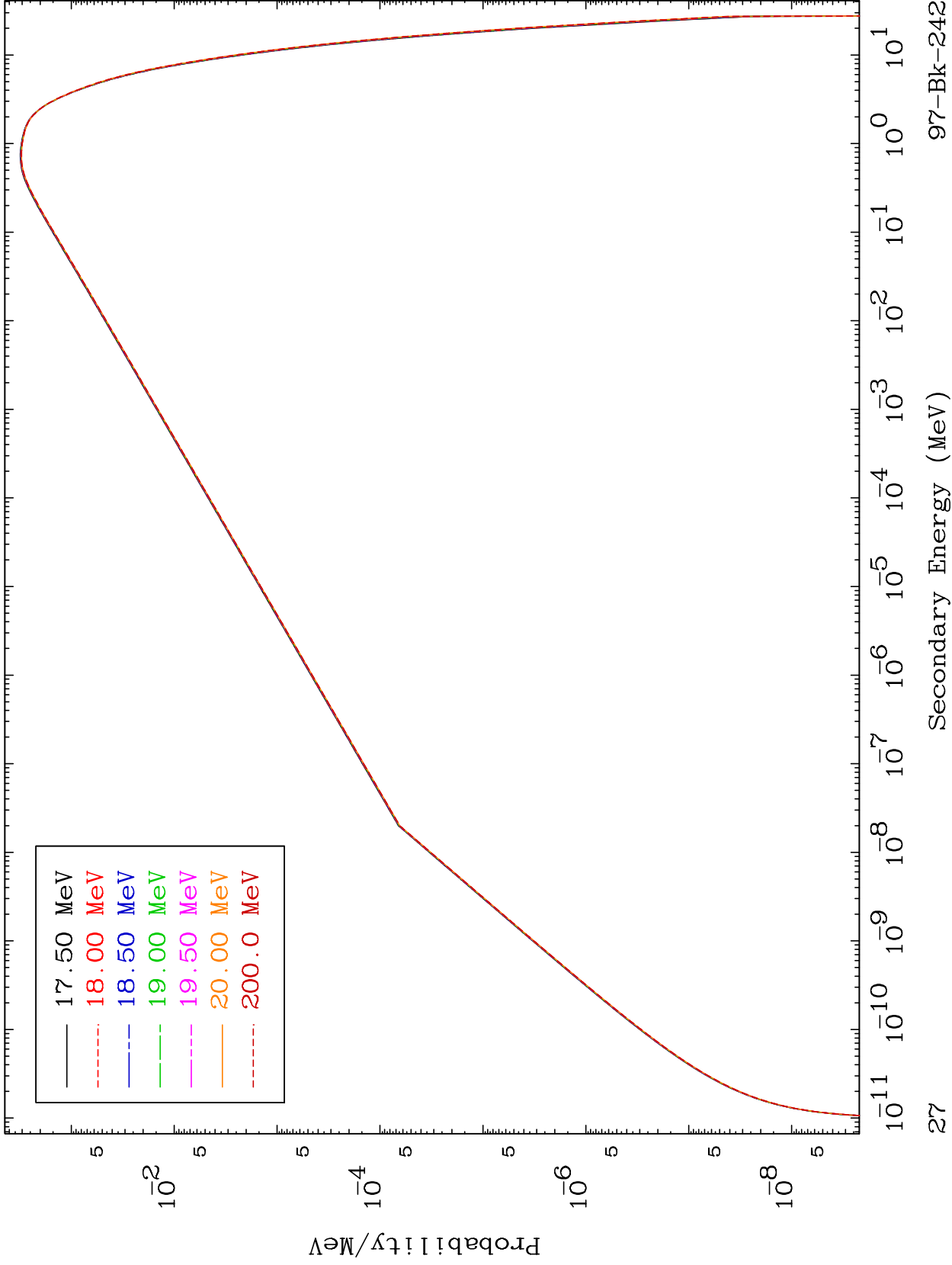
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### Fission Energy Distributions

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(n,n') p  $\alpha$

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Radionuclide Production Cross Section

