

Program EVALPLOT
(Version 2021-1)

by

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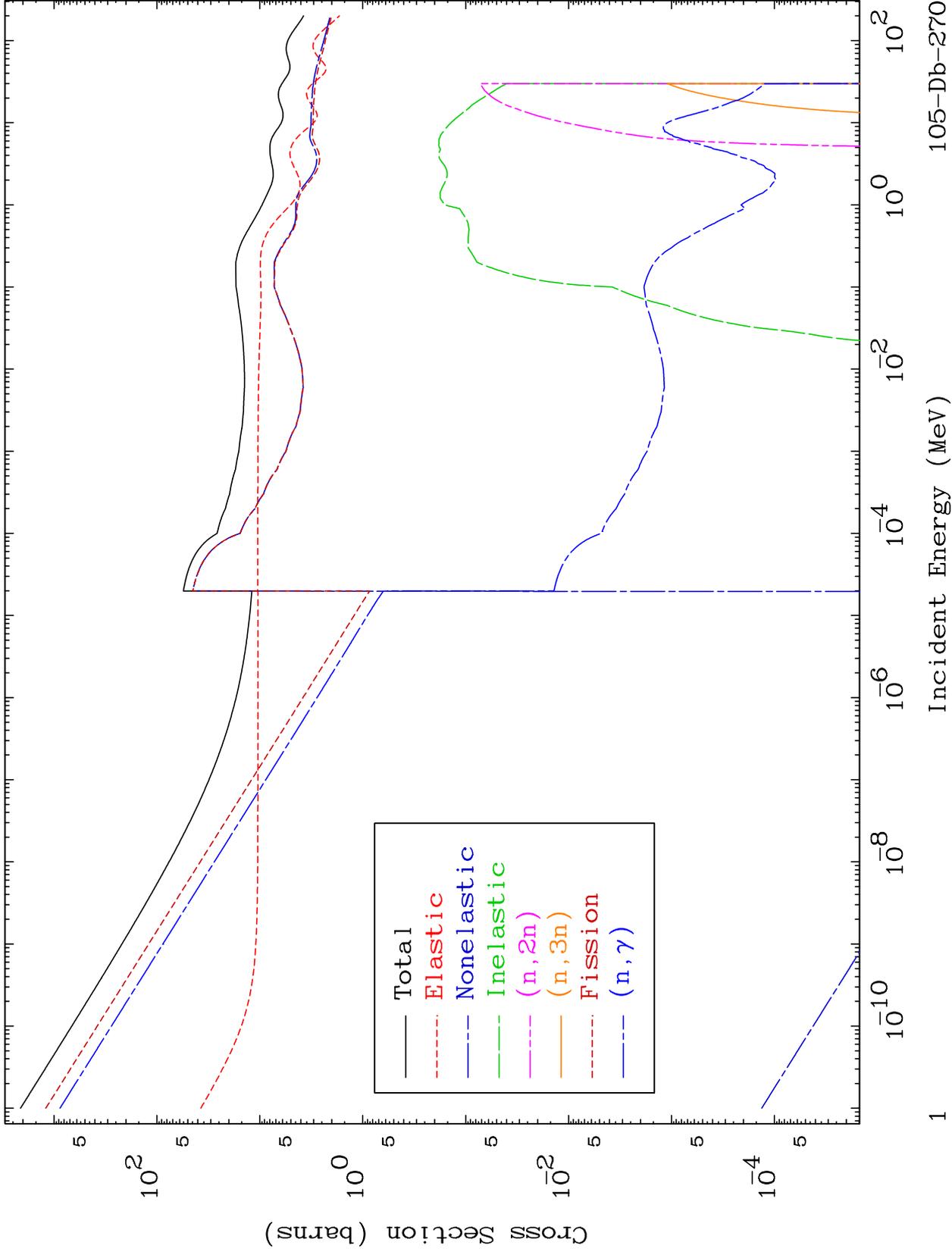
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Press Mouse Button to Start

MAT 570

Neutron Major
293 Kelvin Cross Sections

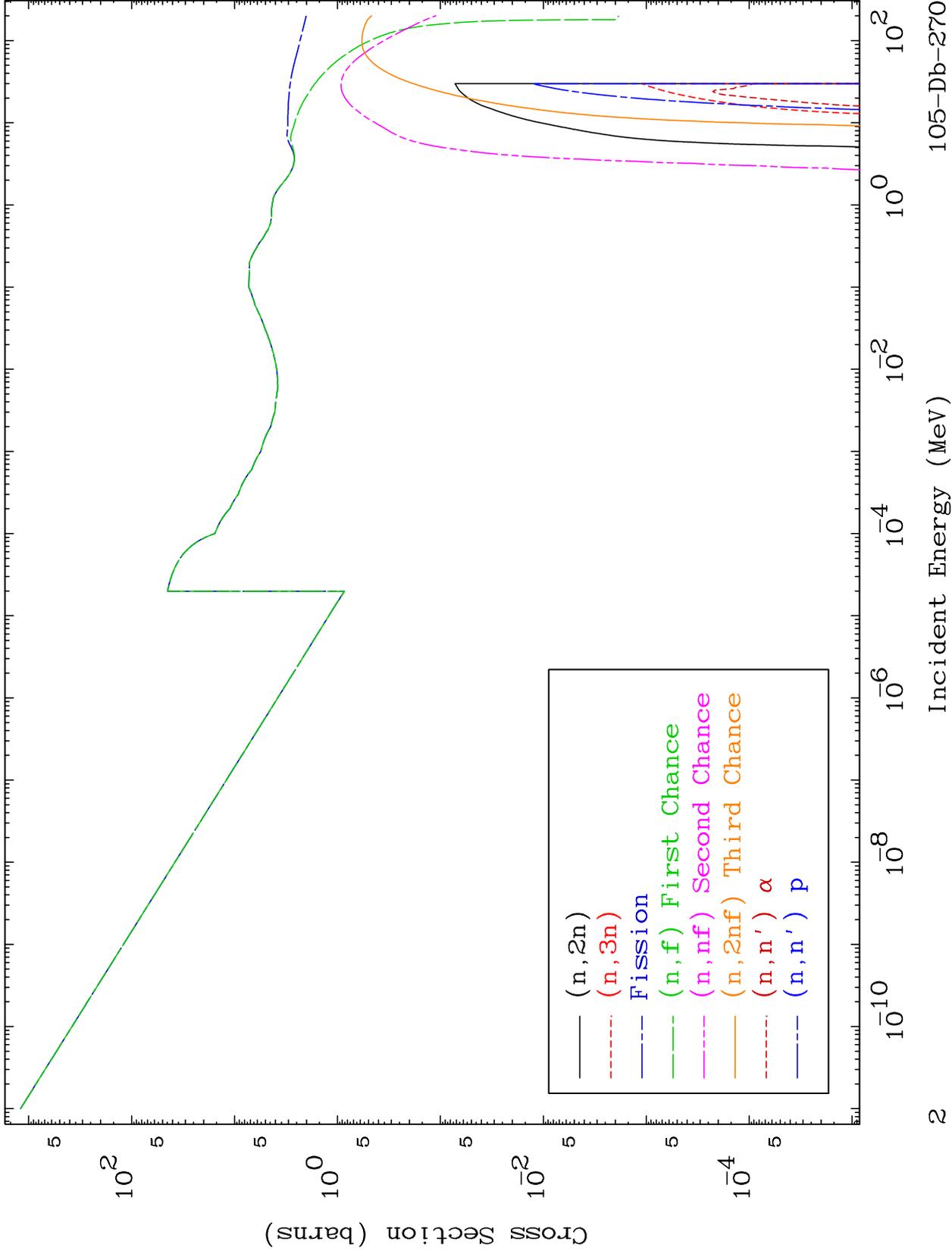
105-Db-270



MAT 570

Neutron Absorption
293 Kelvin Cross Sections

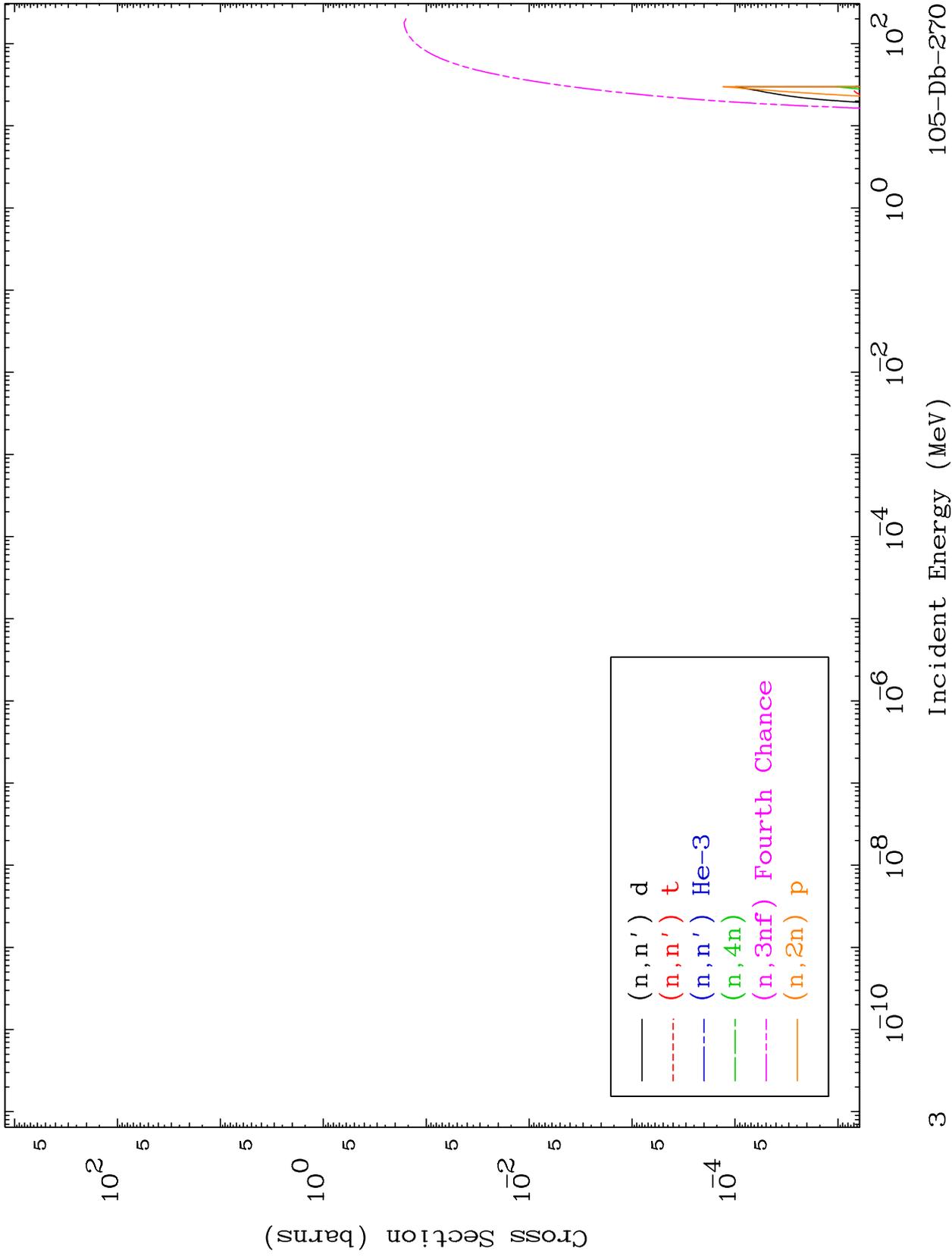
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MAT 570

Neutron Absorption
293 Kelvin Cross Sections

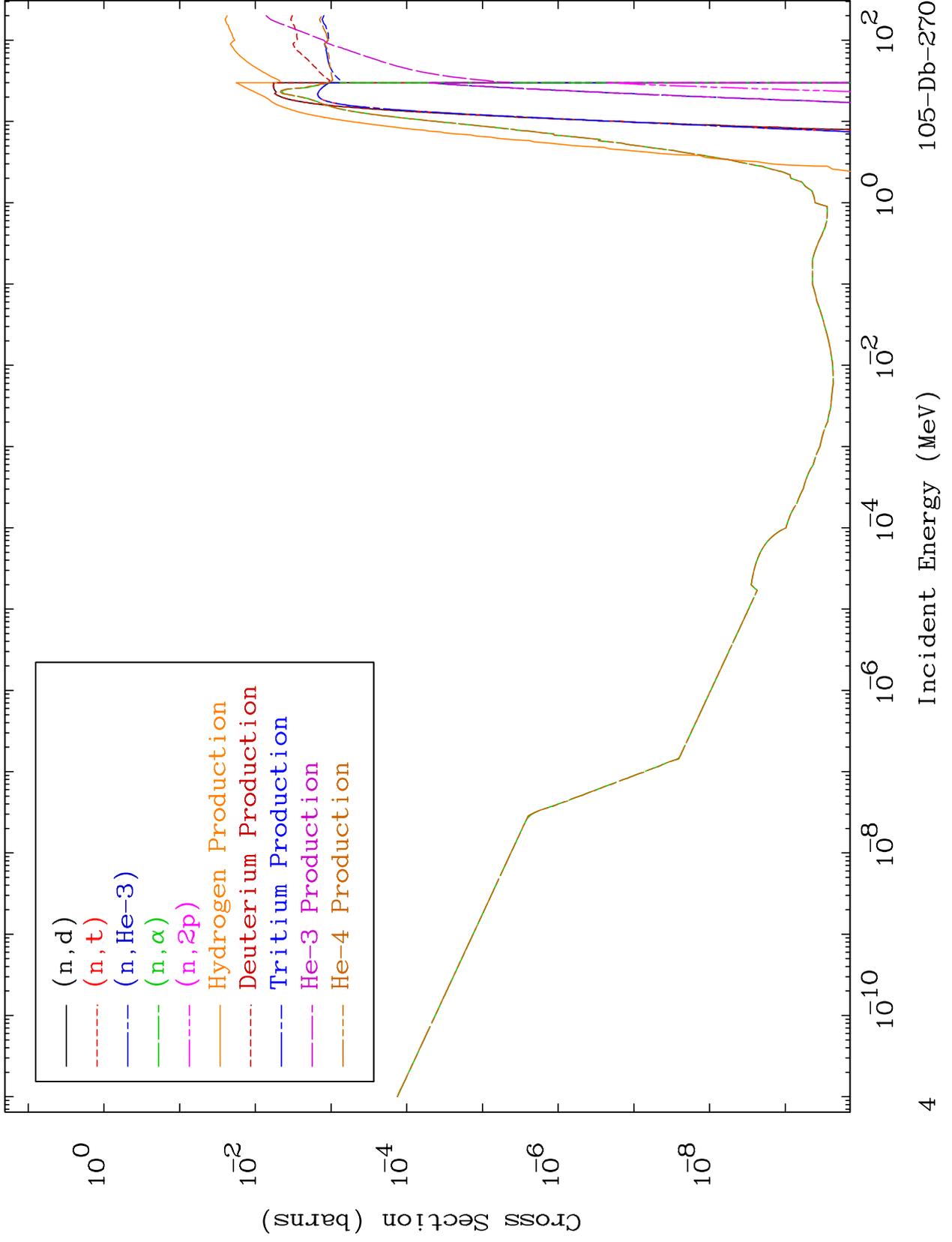
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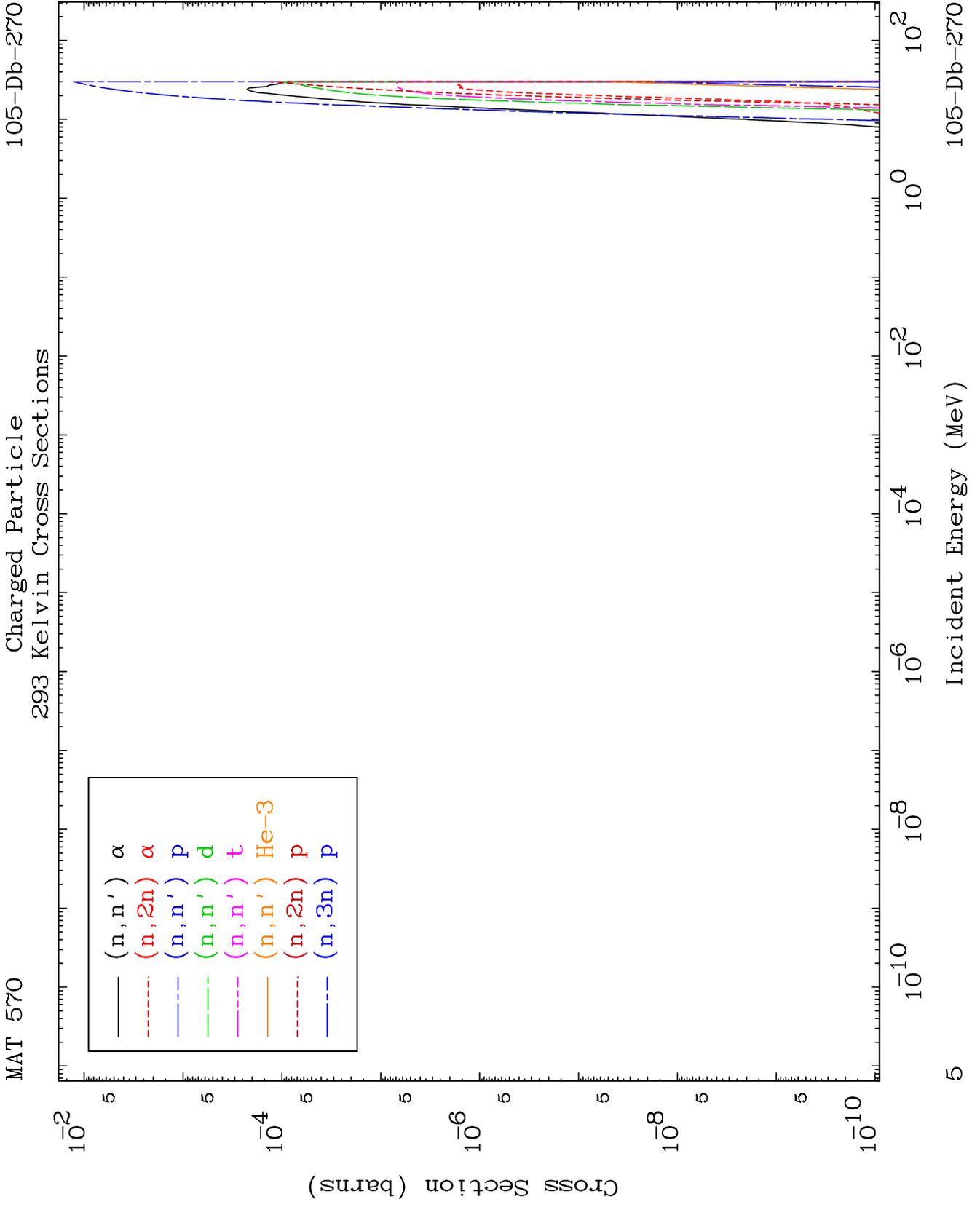


MAT 570

Neutron Absorption
293 Kelvin Cross Sections

105-Db-270

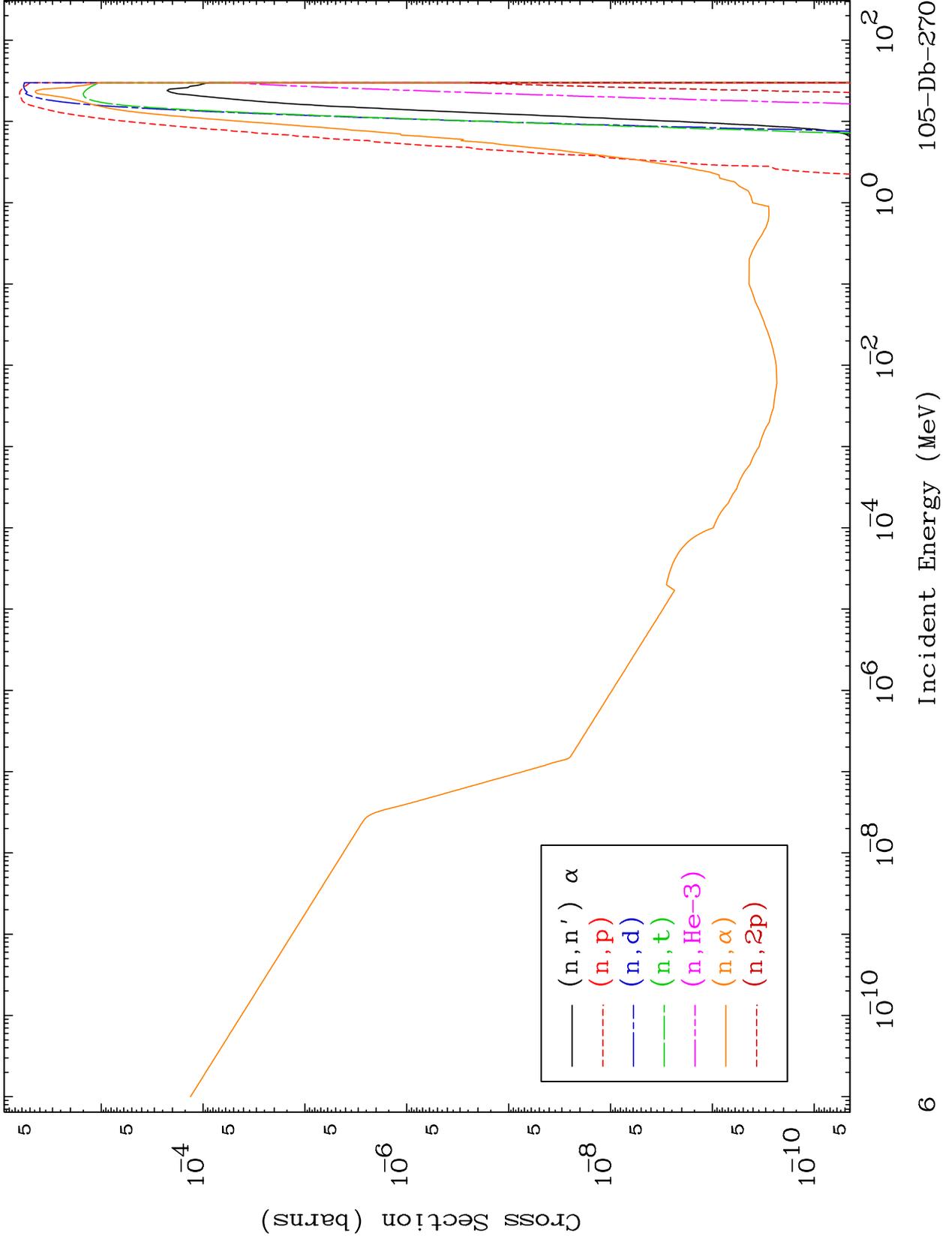




MAT 570

Charged Particle
293 Kelvin Cross Sections

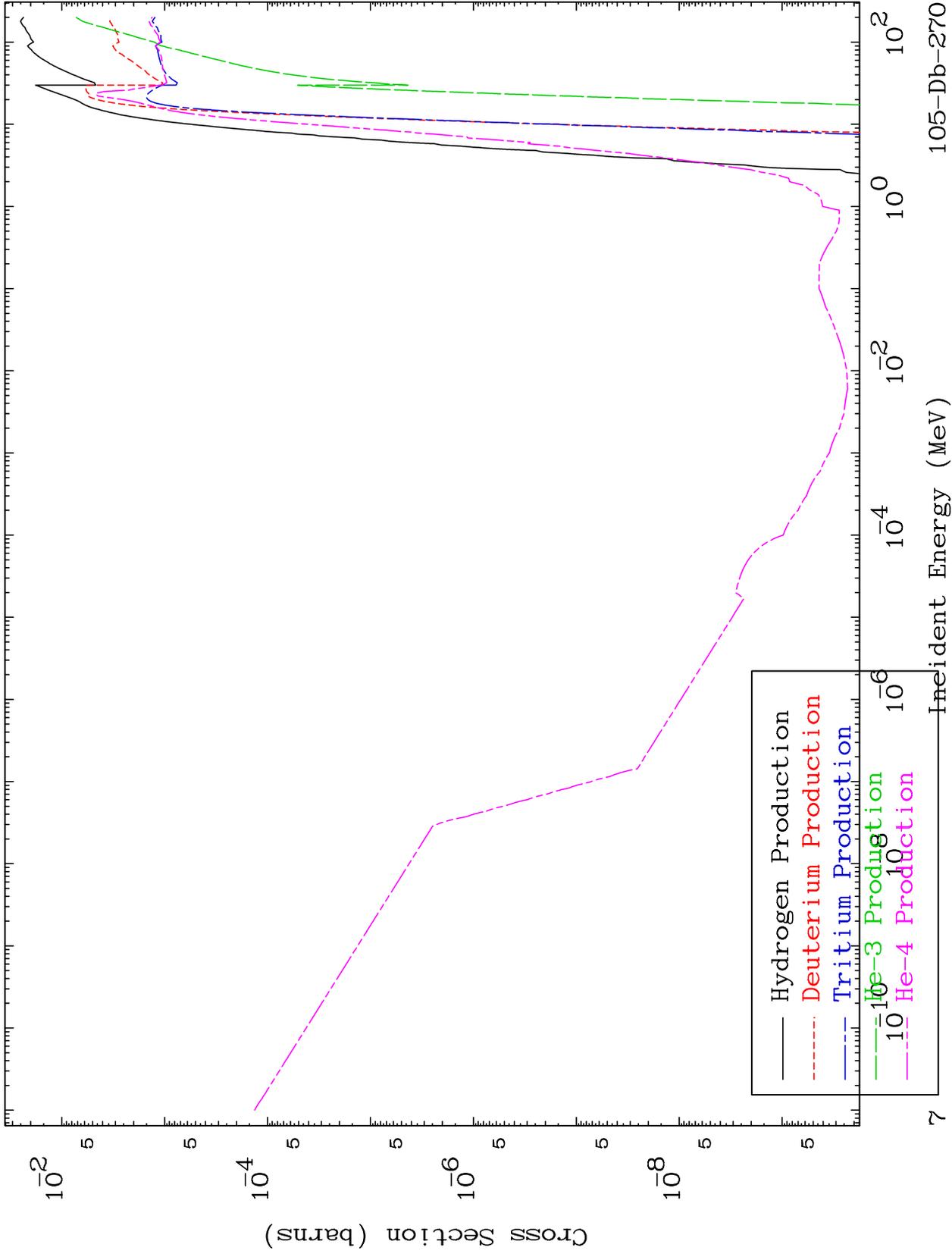
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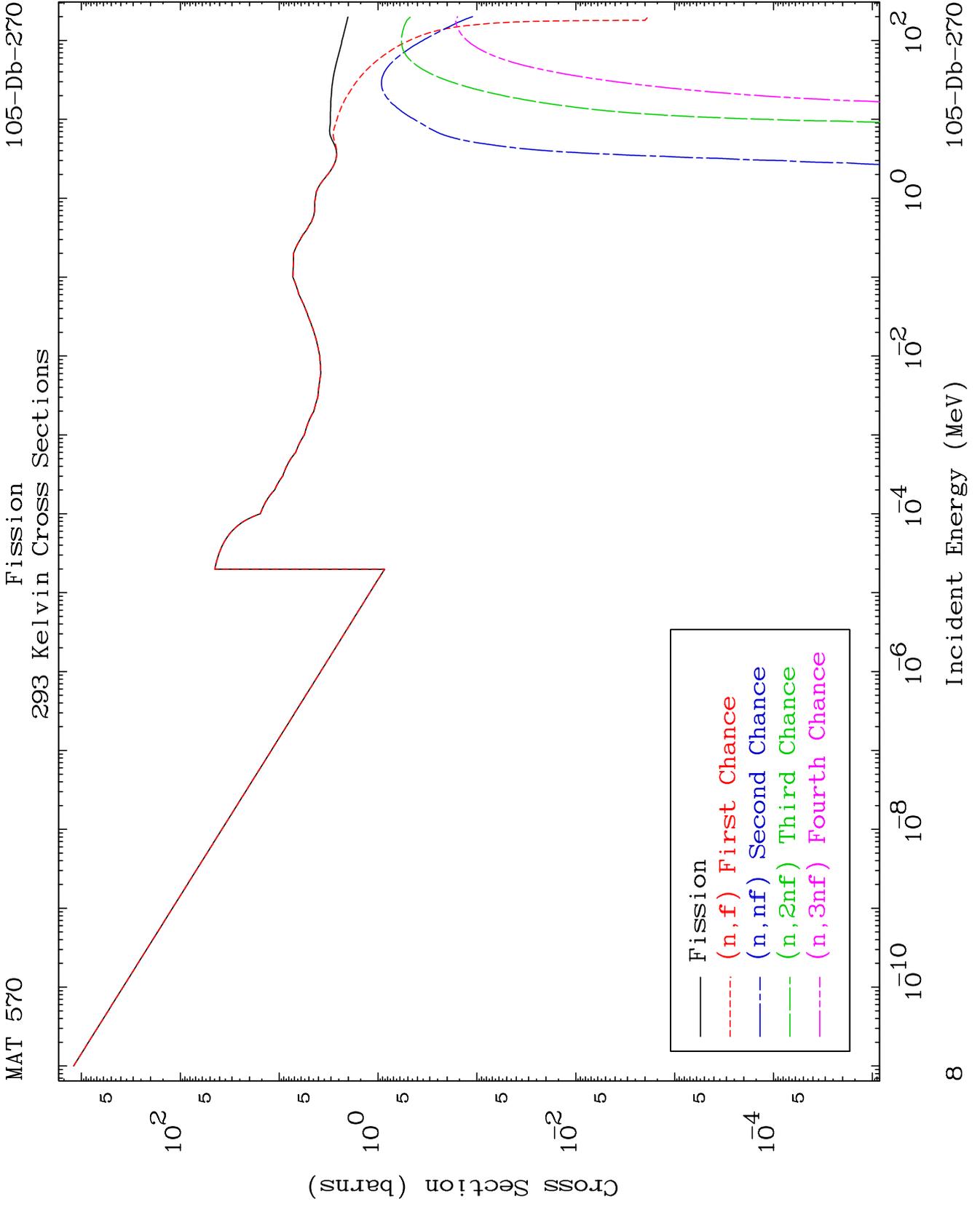


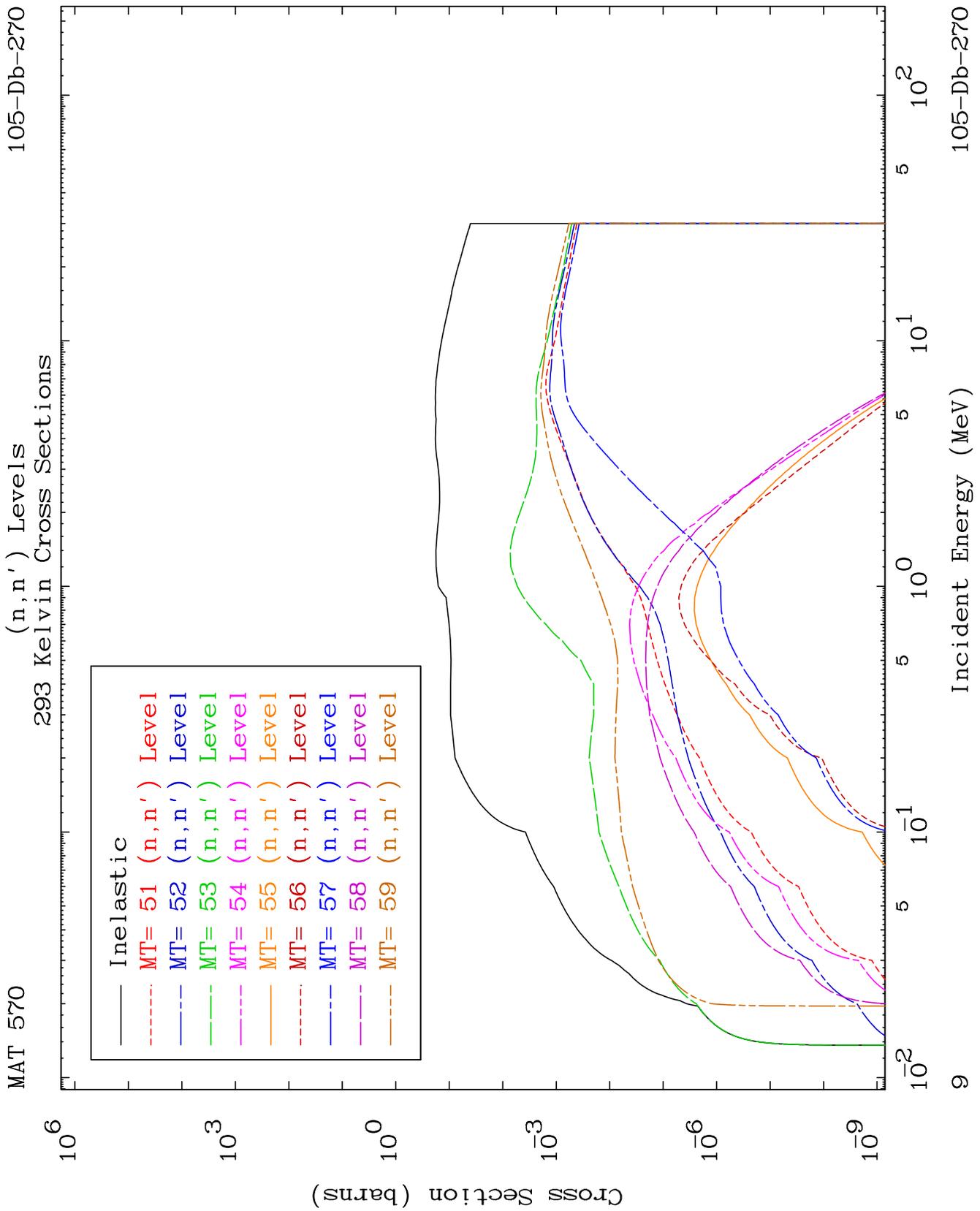
MAT 570

Particle Production
293 Kelvin Cross Sections

105-Db-270



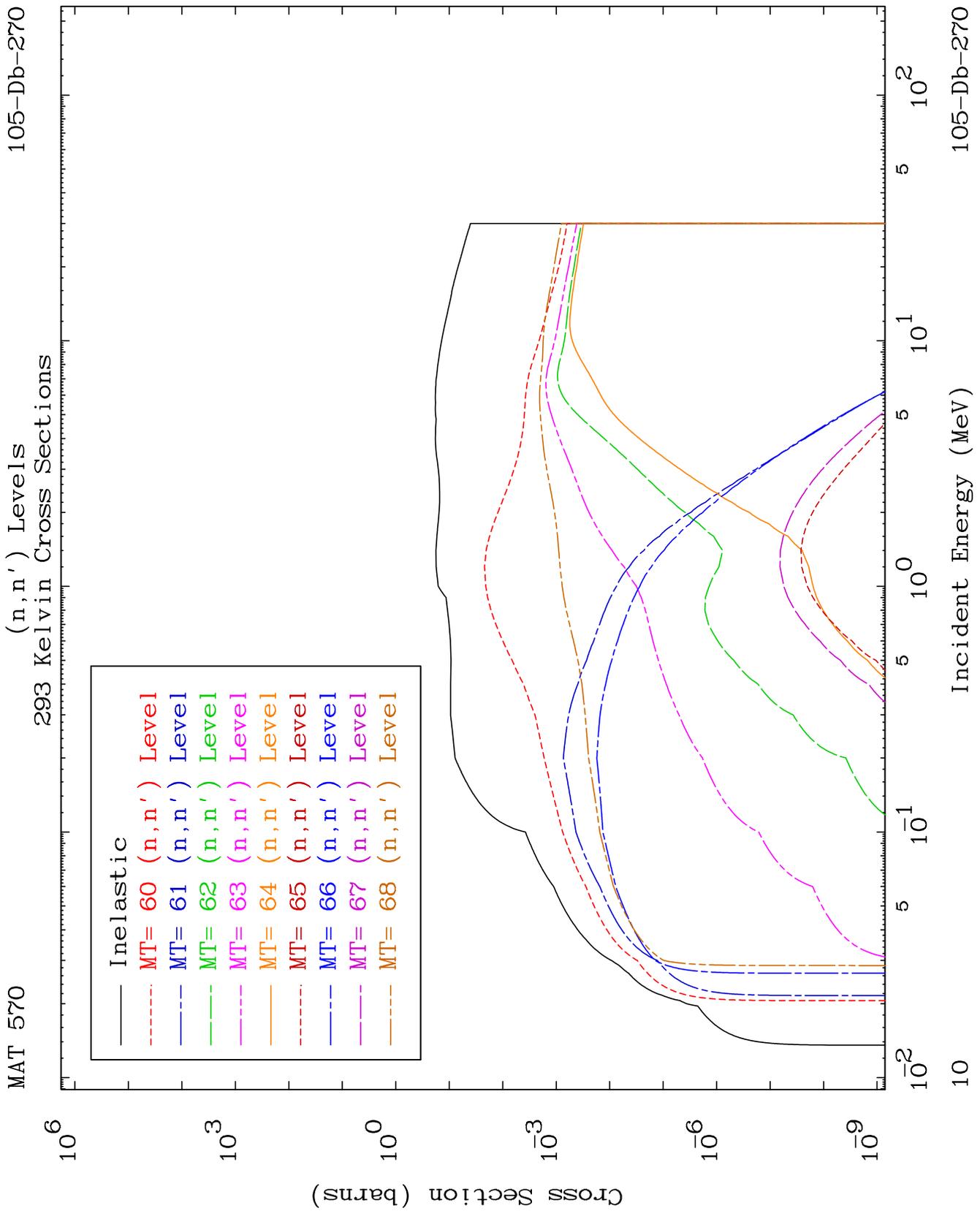




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Incident Energy (MeV)

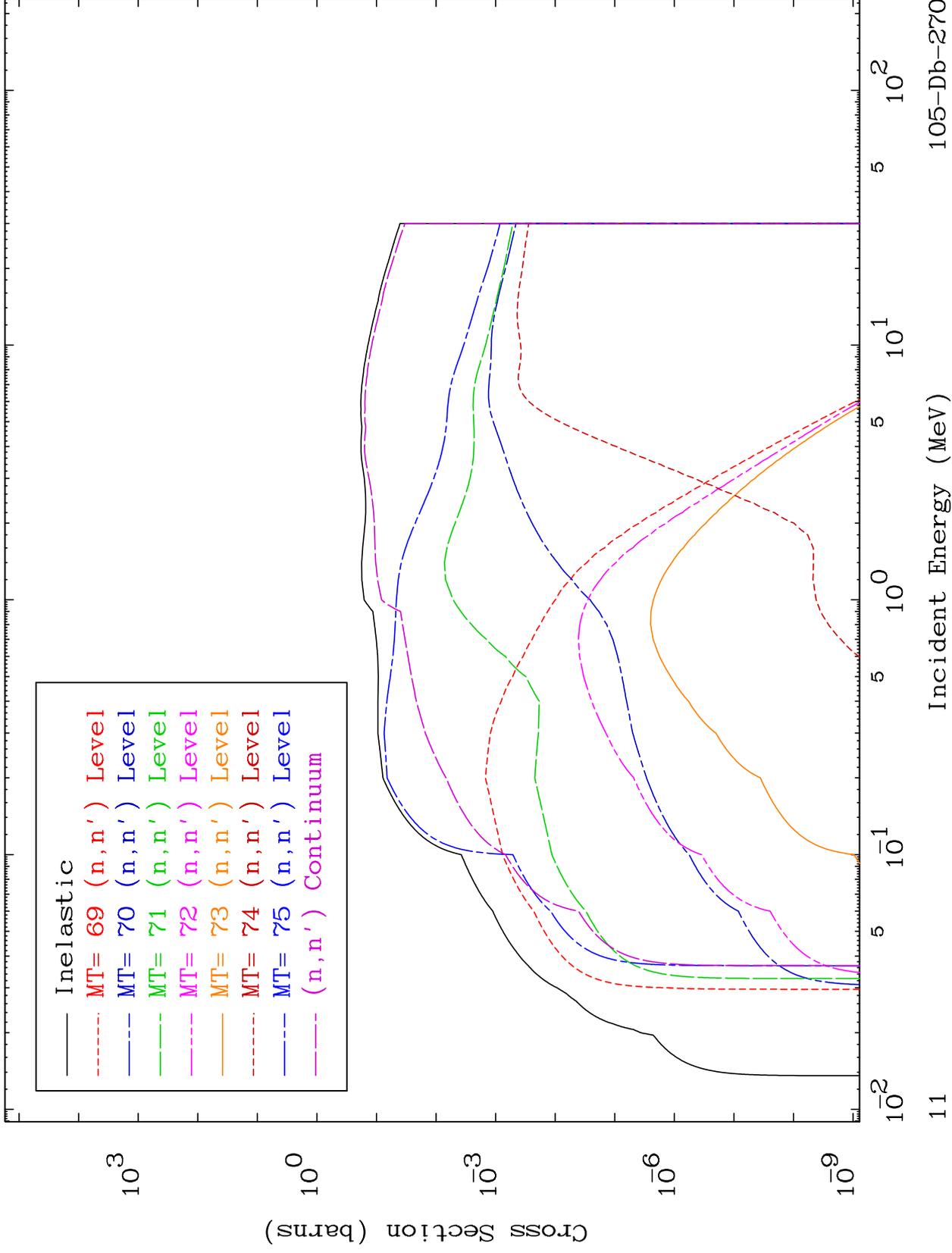
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MAT 570

(n,n') Levels
293 Kelvin Cross Sections

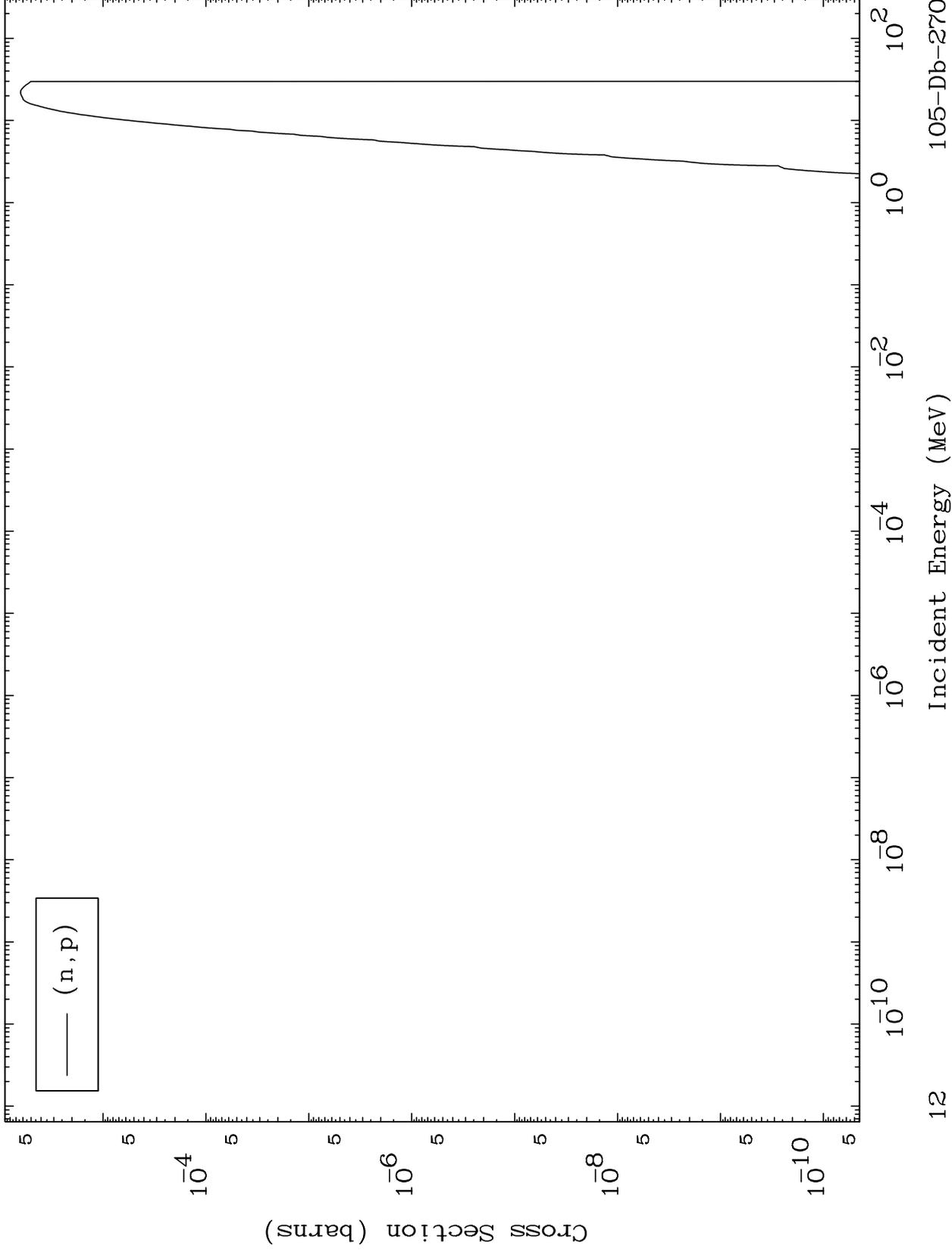
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MAT 570

(n,p) Levels
293 Kelvin Cross Sections

105-Db-270

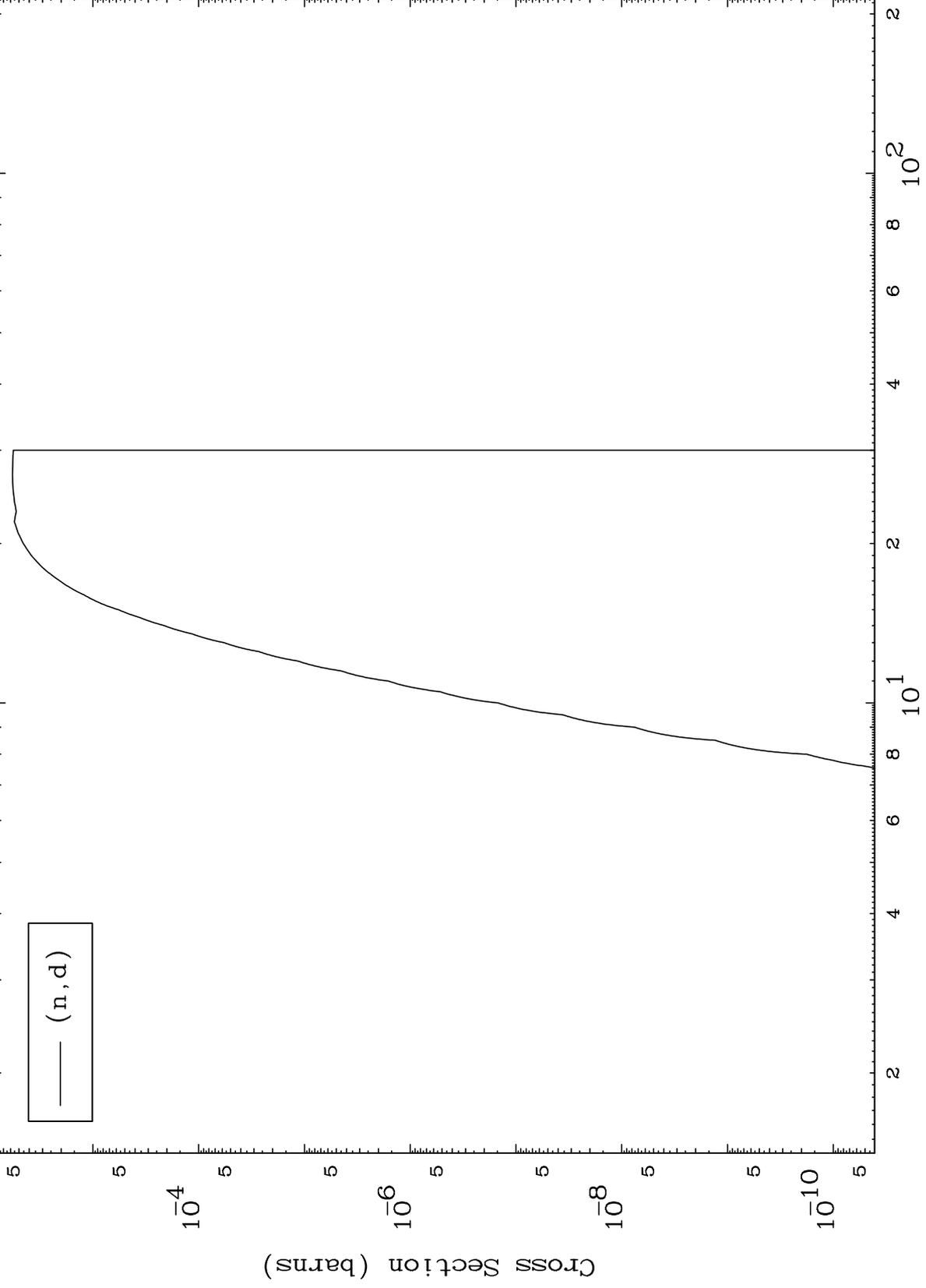


(n,p)

MAT 570

(n,d) Levels
293 Kelvin Cross Sections

105-Db-270



13

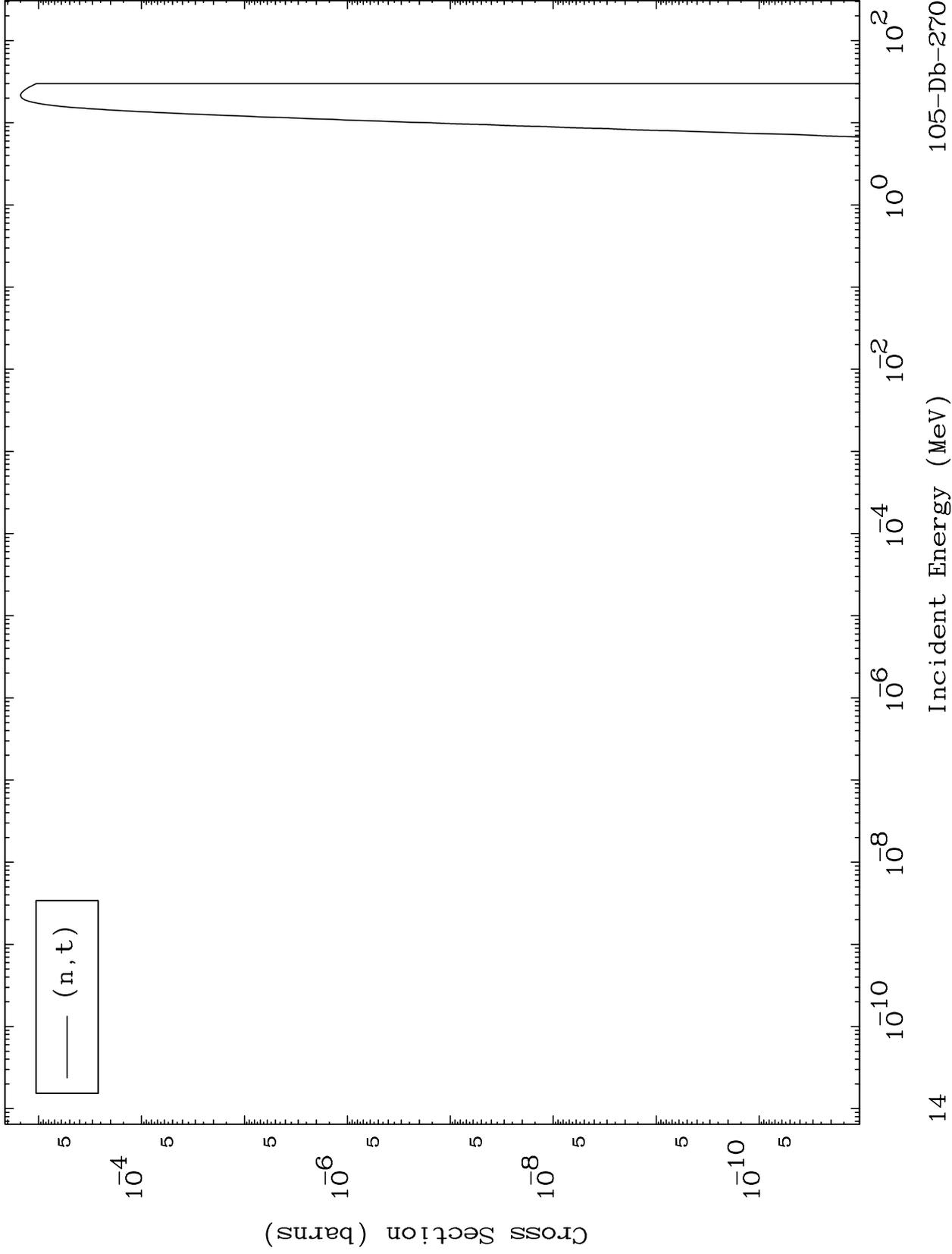
Incident Energy (MeV)

105-Db-270

MAT 570

(n,t) Levels
293 Kelvin Cross Sections

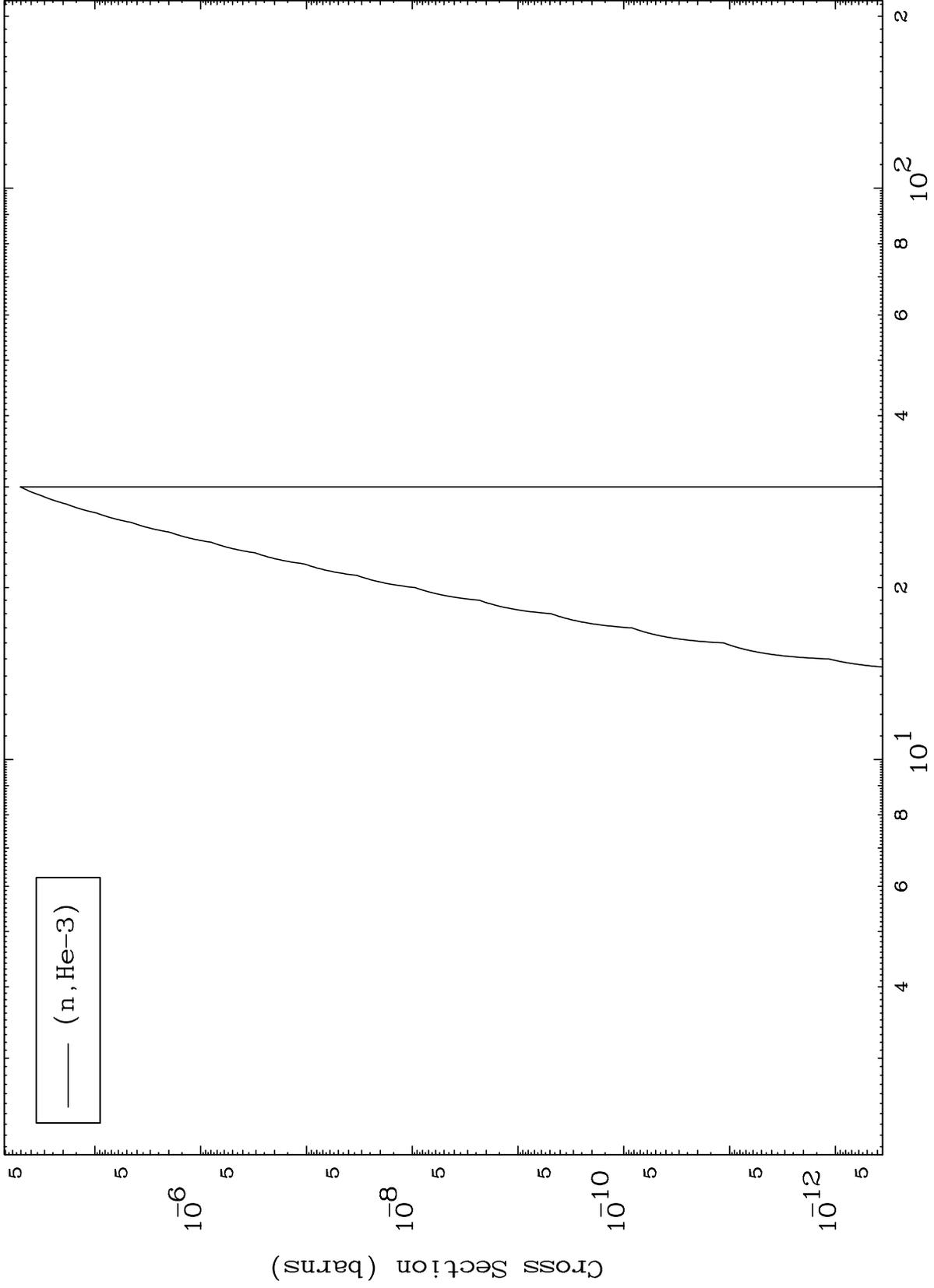
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MAT 570

(n,He3) Levels
293 Kelvin Cross Sections

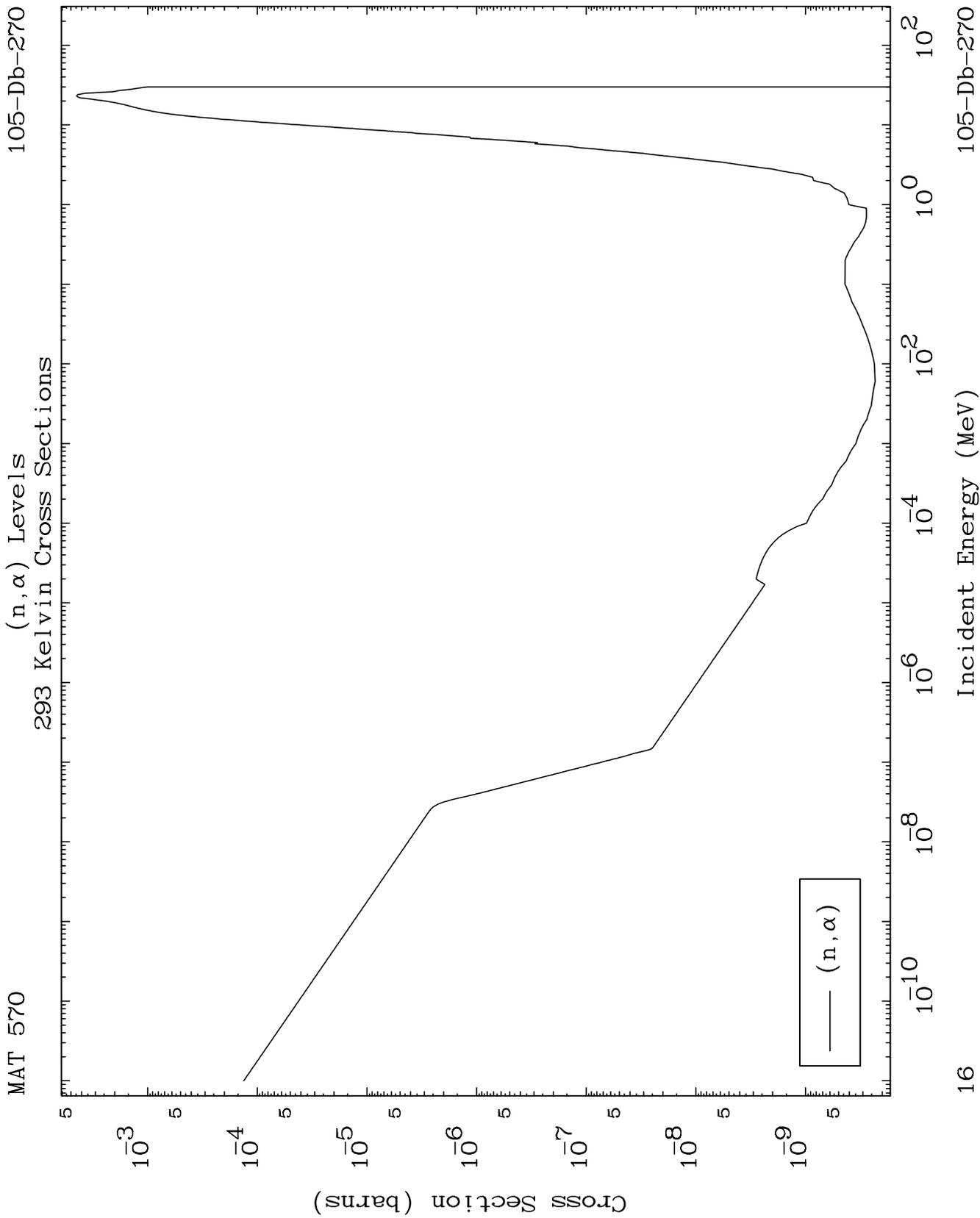
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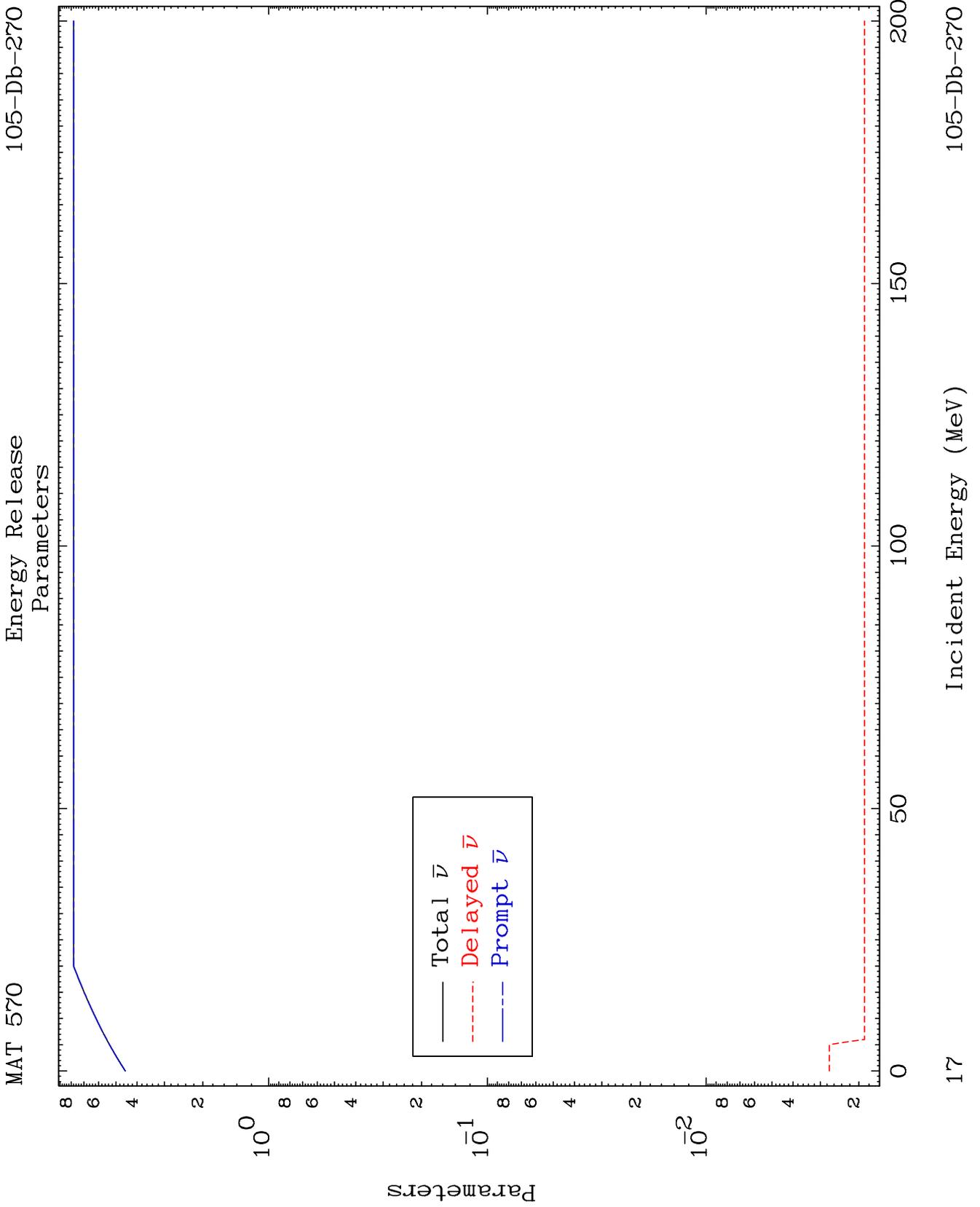


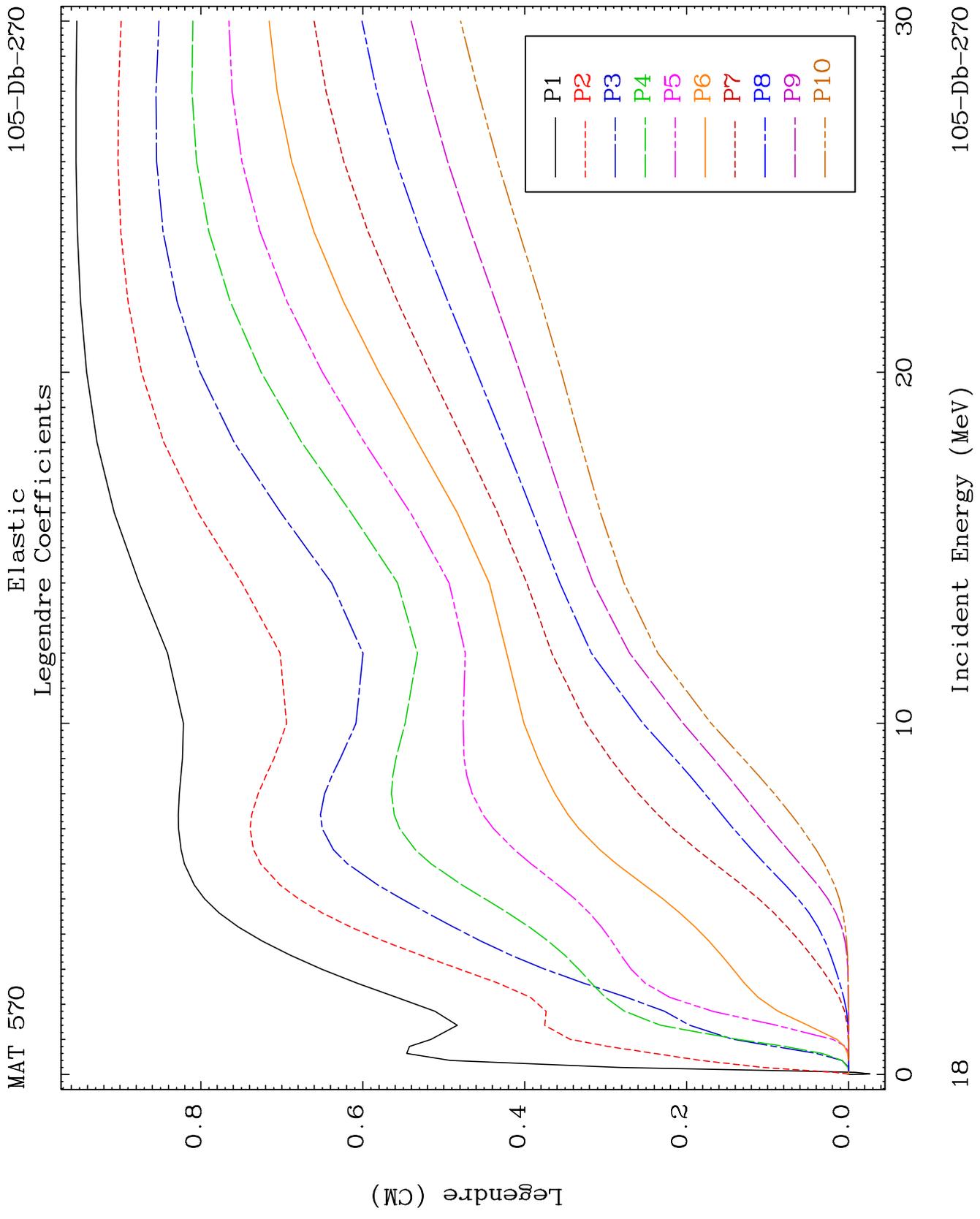
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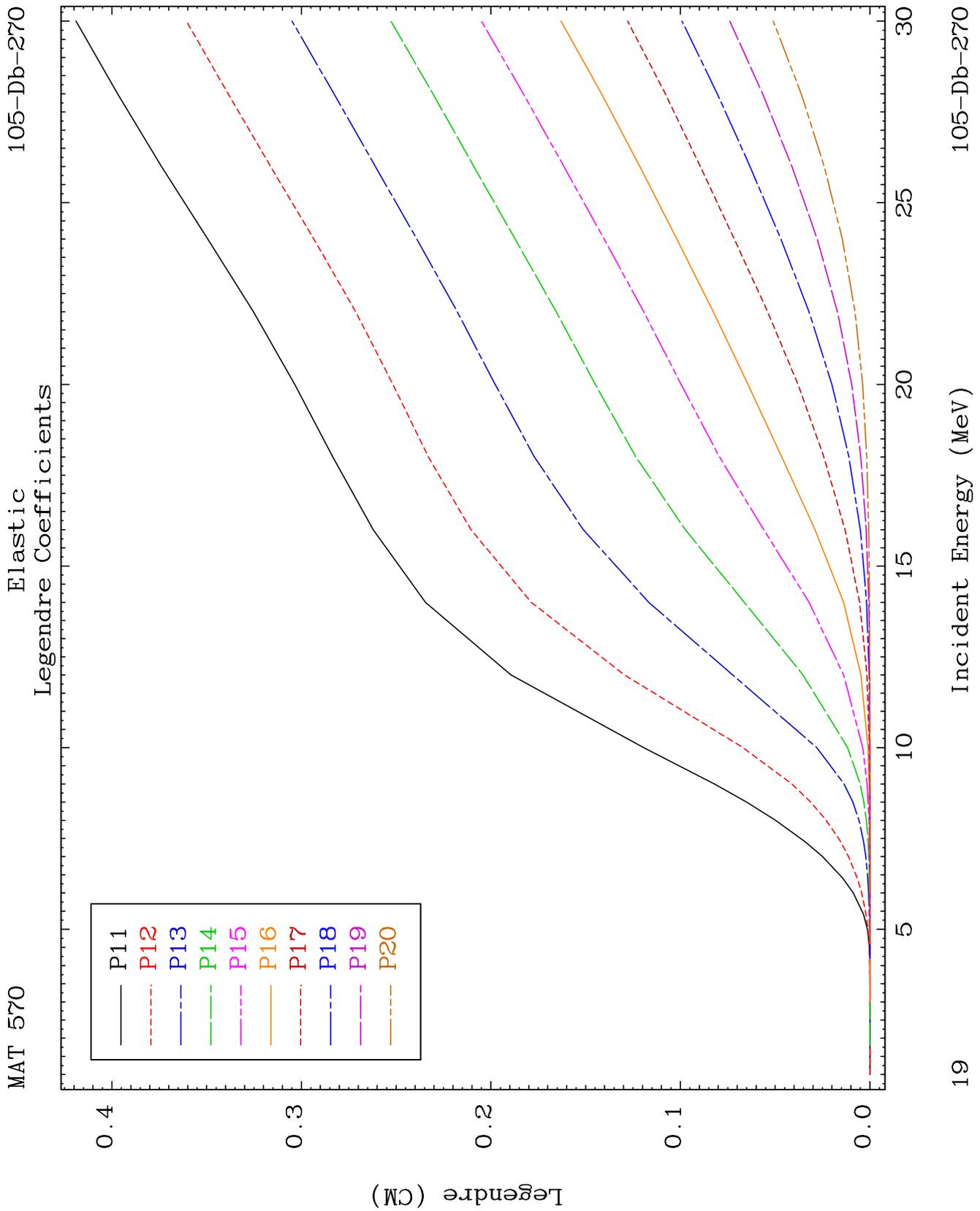
Incident Energy (MeV)

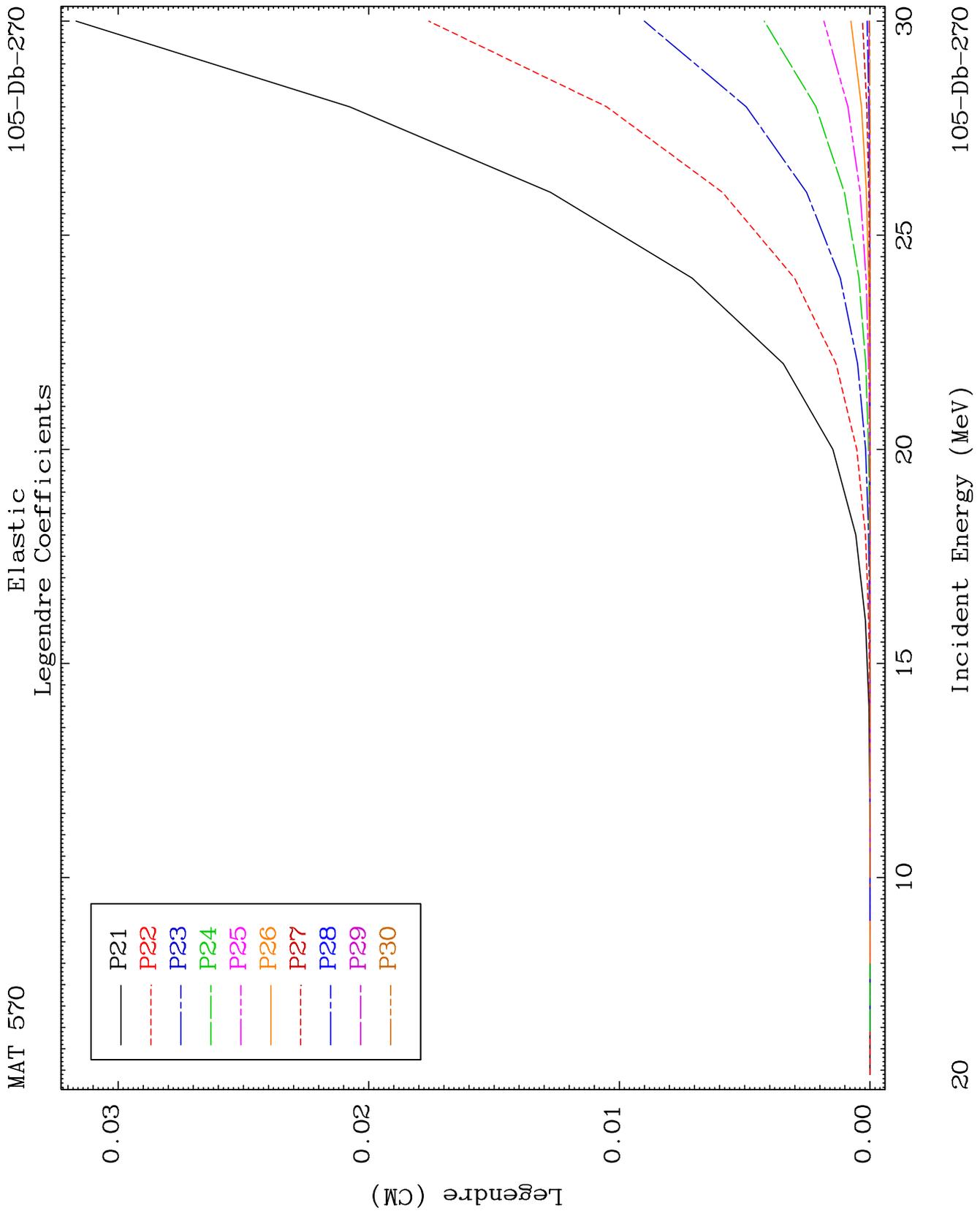
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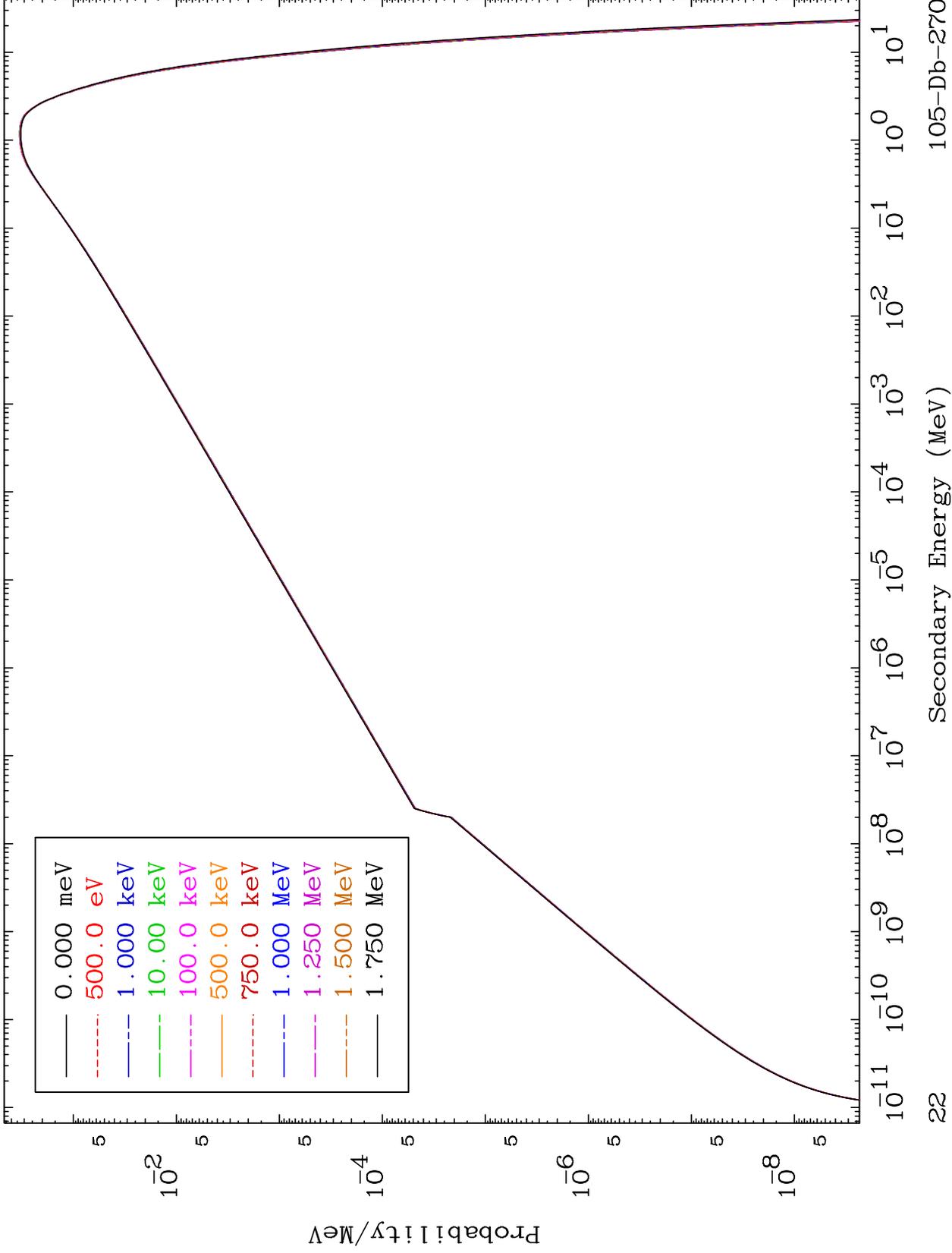




MAT 570

Fission Energy Distributions

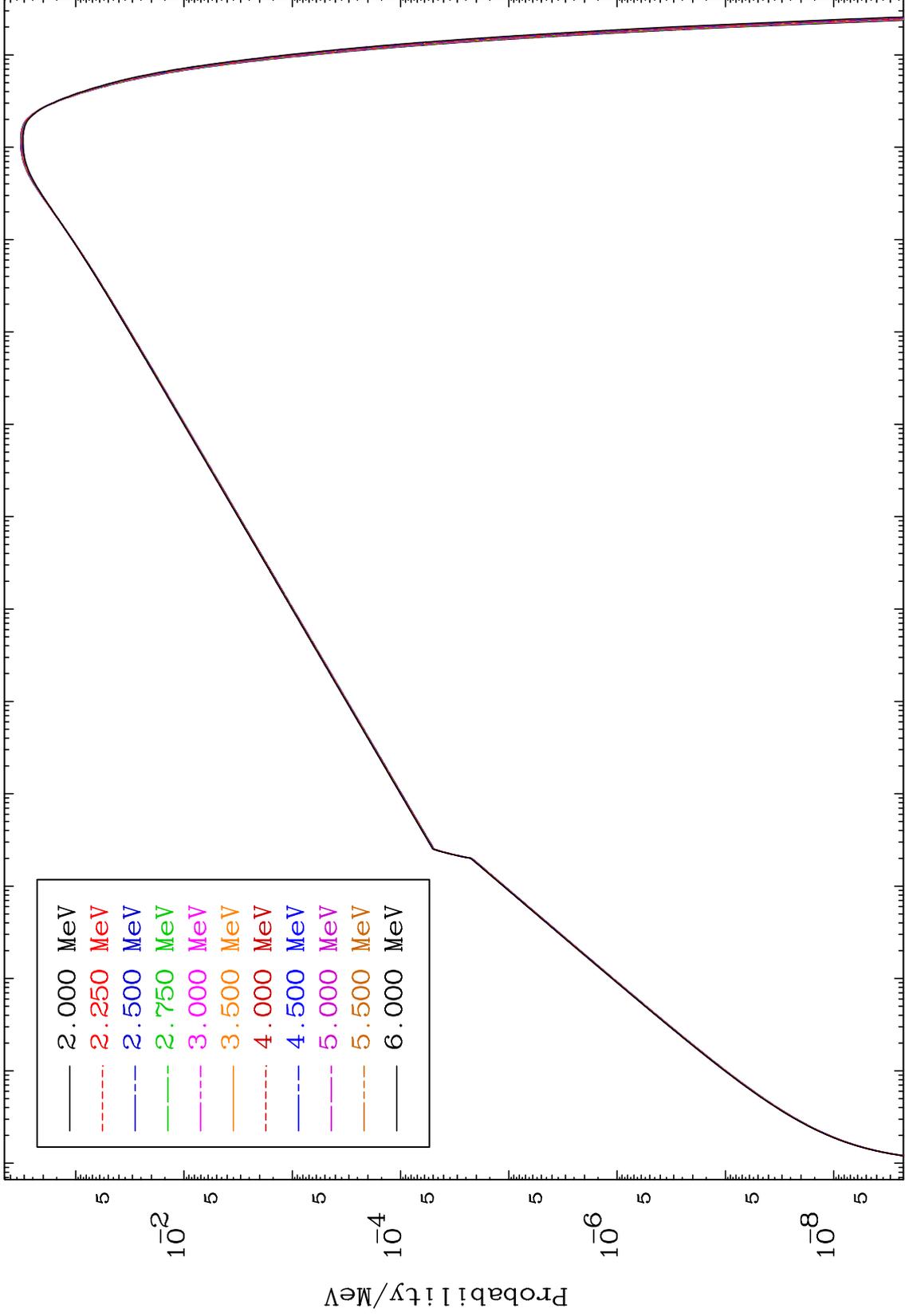
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MAT 570

Fission Energy Distributions

105-Db-270



23

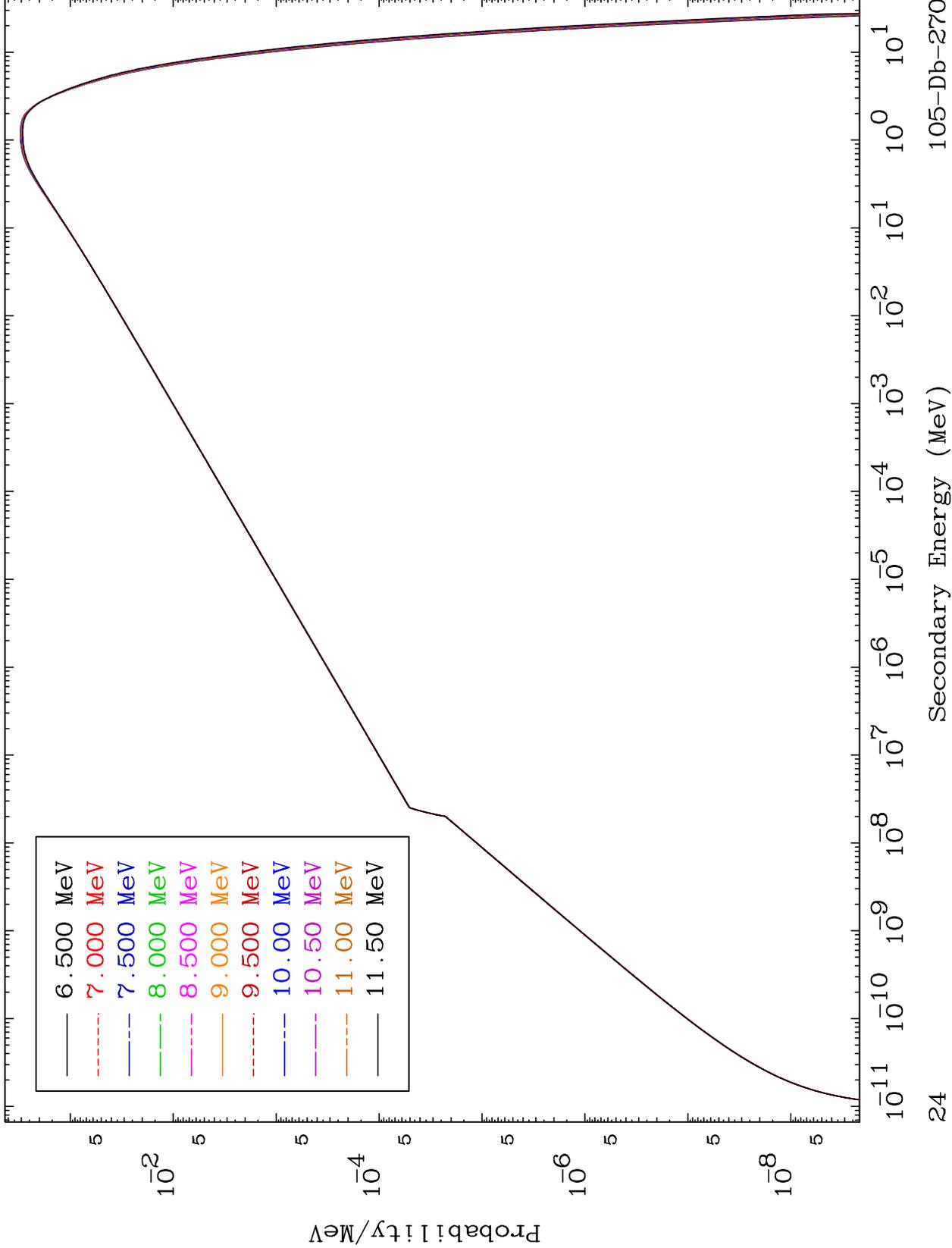
Secondary Energy (MeV)

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MAT 570

Fission Energy Distributions

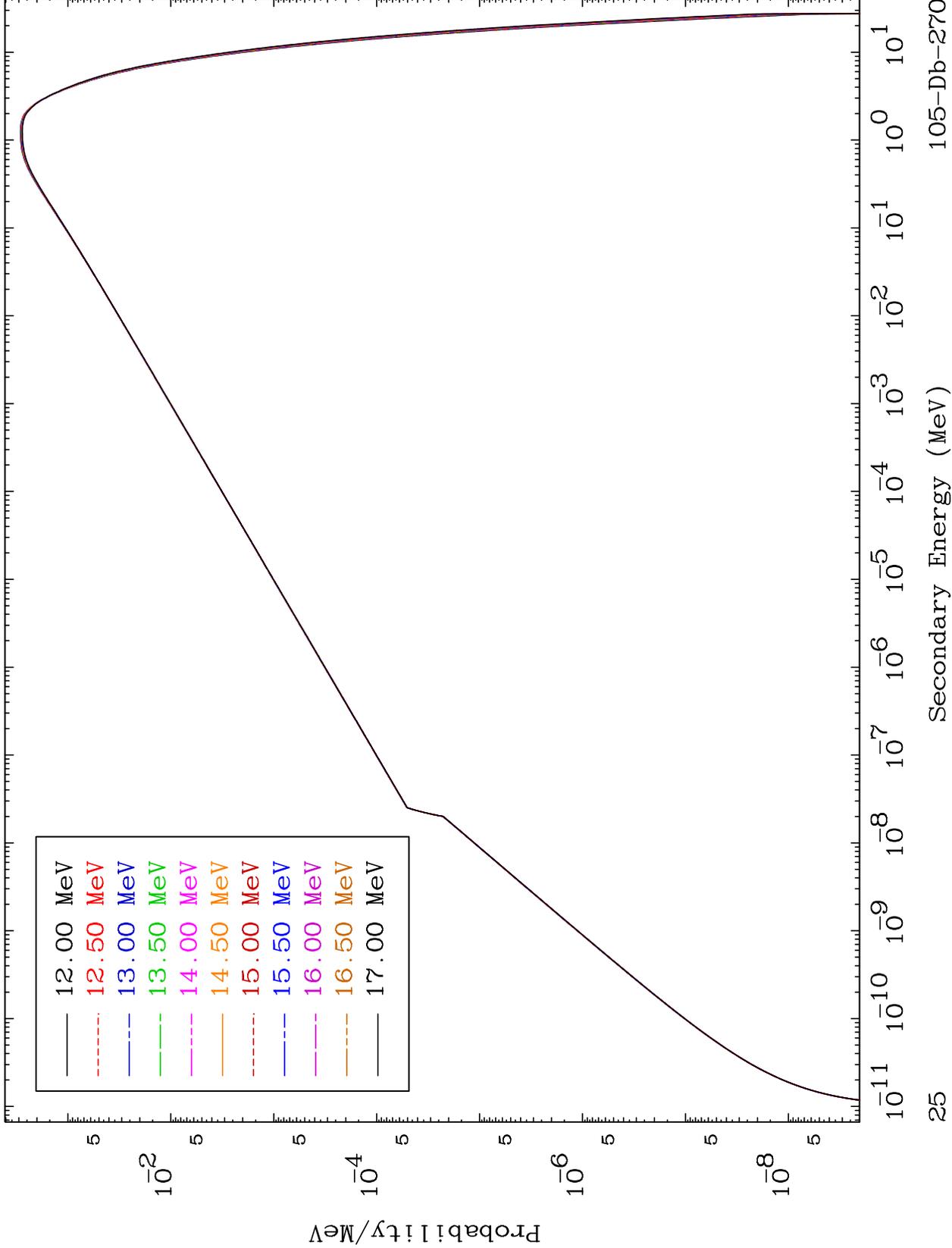
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MAT 570

Fission Energy Distributions

105-Db-270



MAT 570

Fission Energy Distributions

105-Db-270

