

Program EVALPLOT
(Version 2021-1)

by

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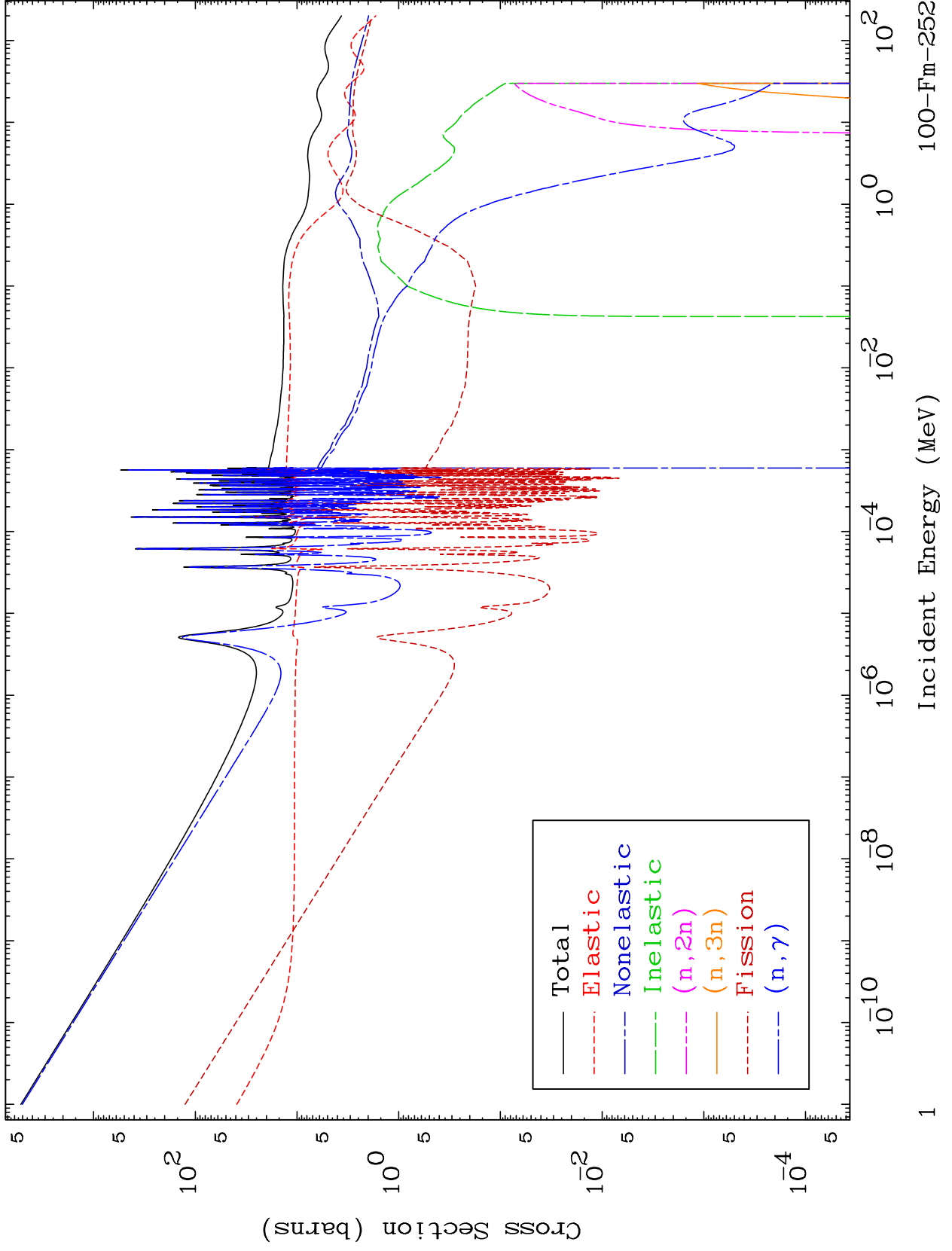
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9933

Neutron Major
293 Kelvin Cross Sections

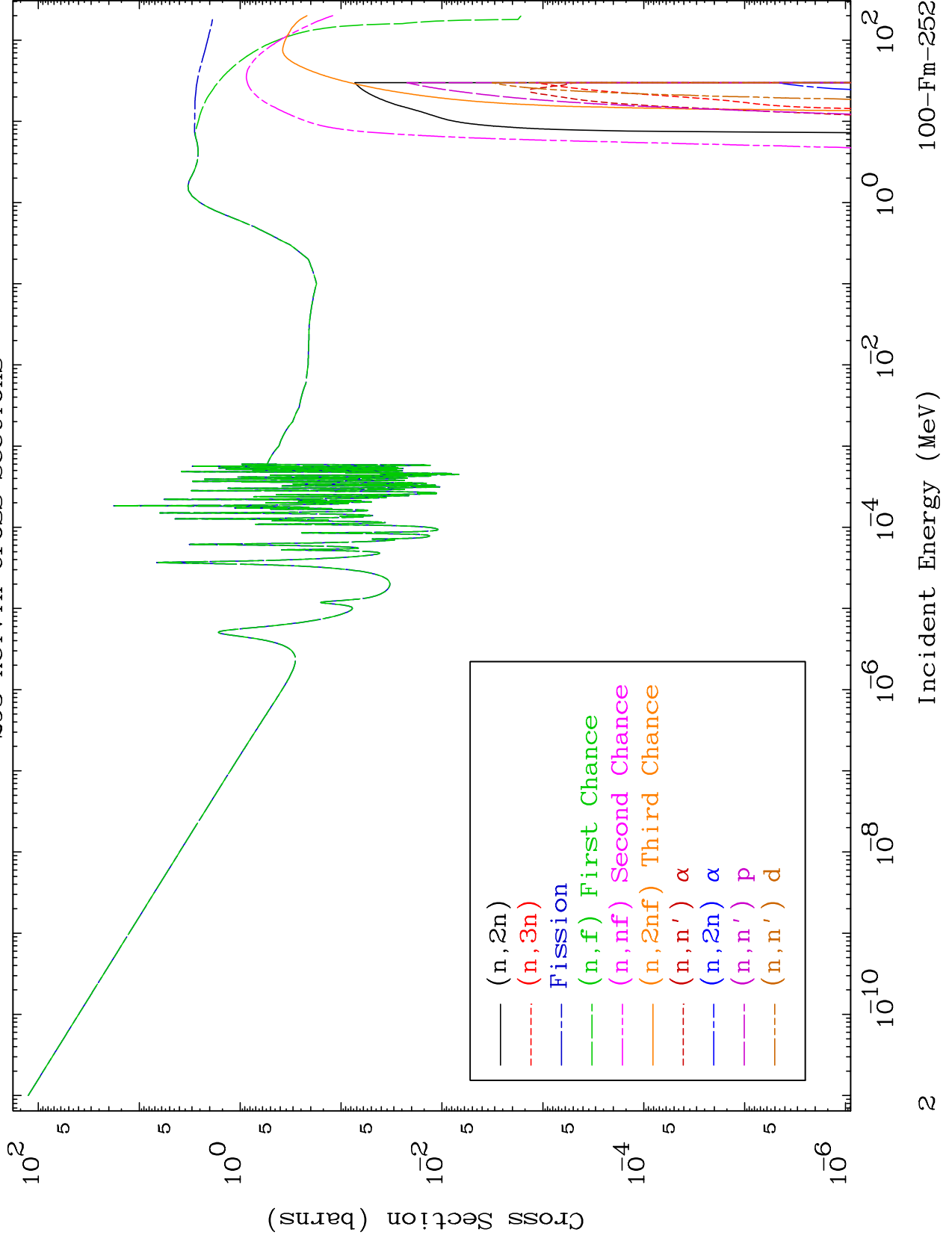
100-Fm-252



MAT 9933

Neutron Absorption
293 Kelvin Cross Sections

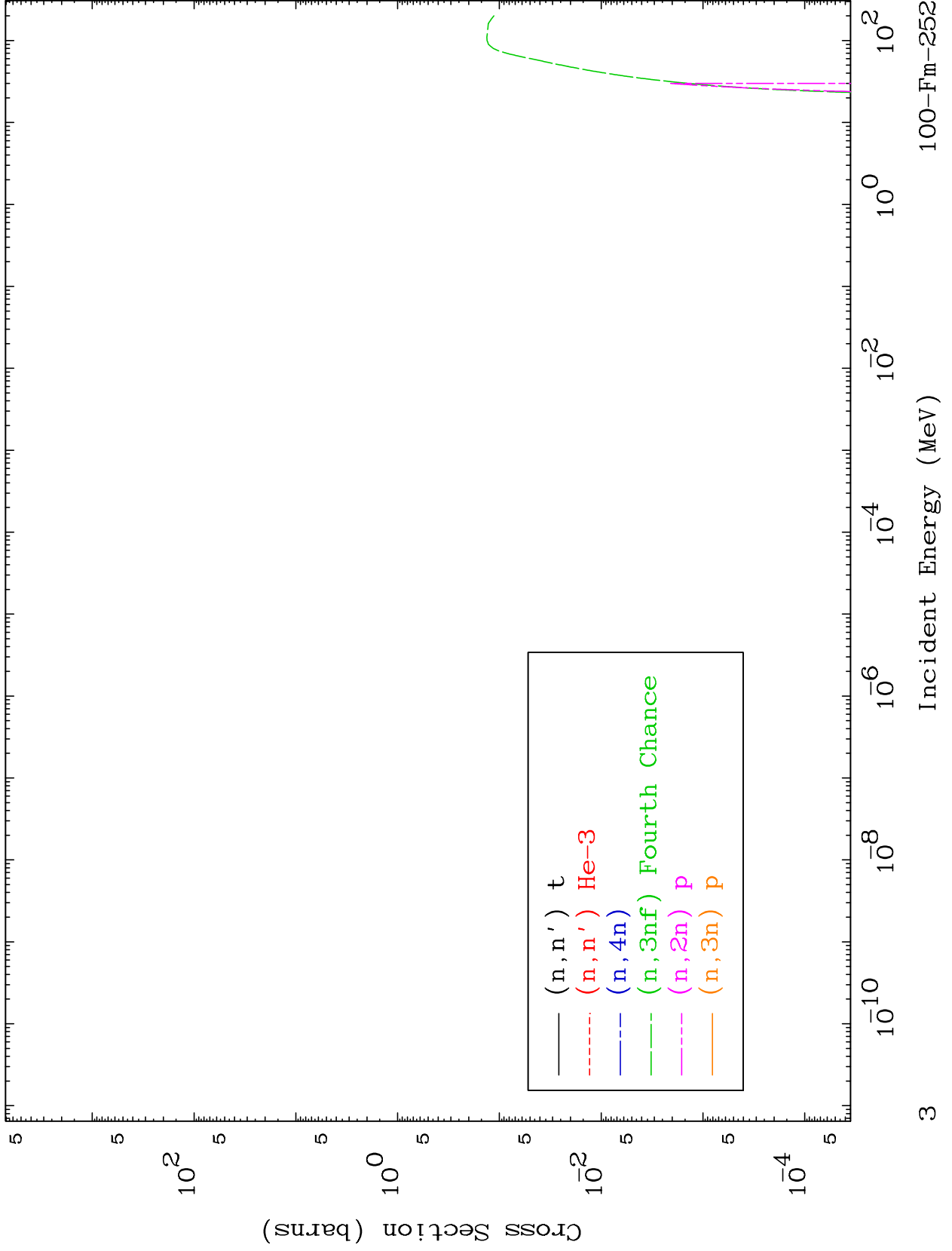
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MAT 9933

Neutron Absorption
293 Kelvin Cross Sections

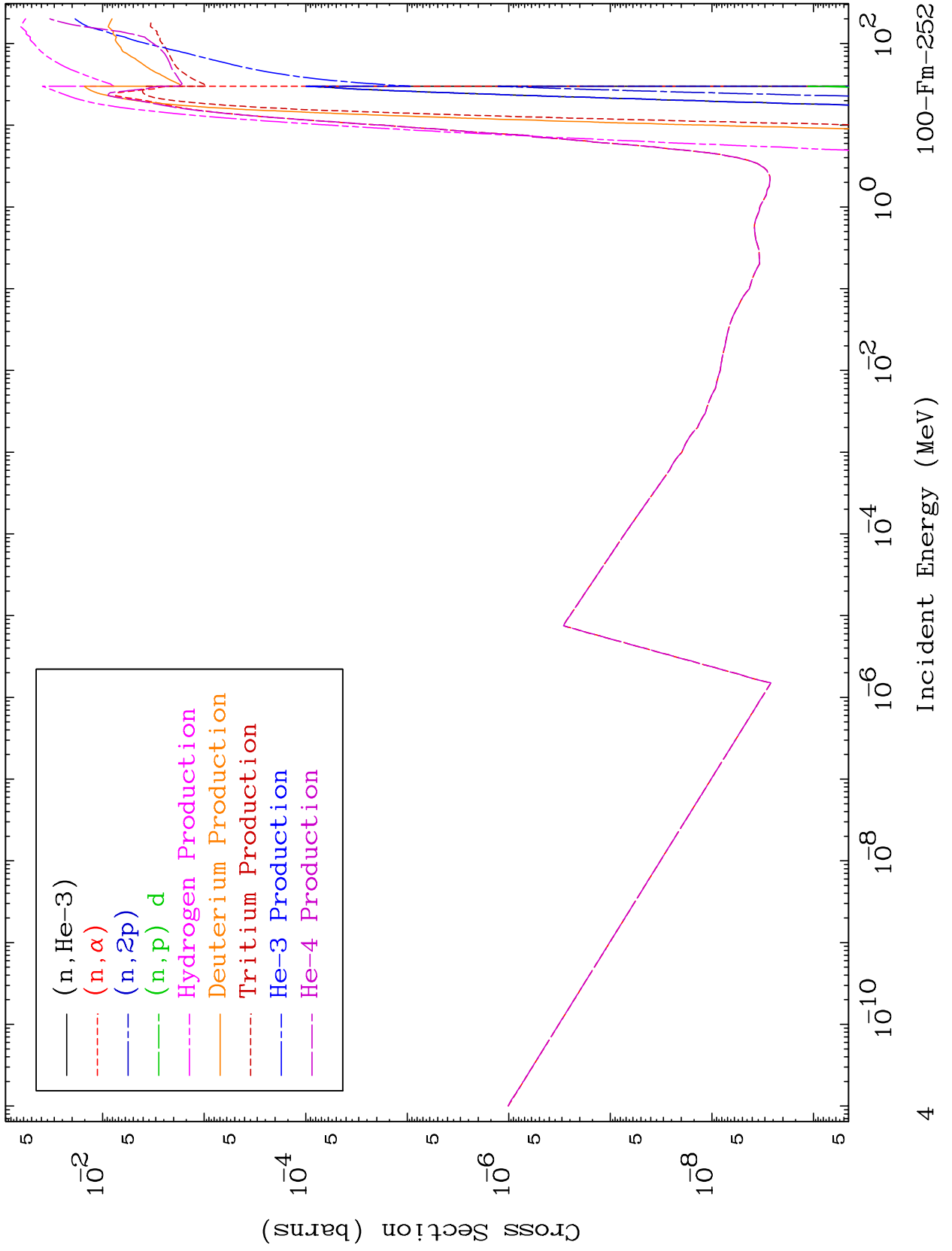
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MAT 9933

Neutron Absorption
293 Kelvin Cross Sections

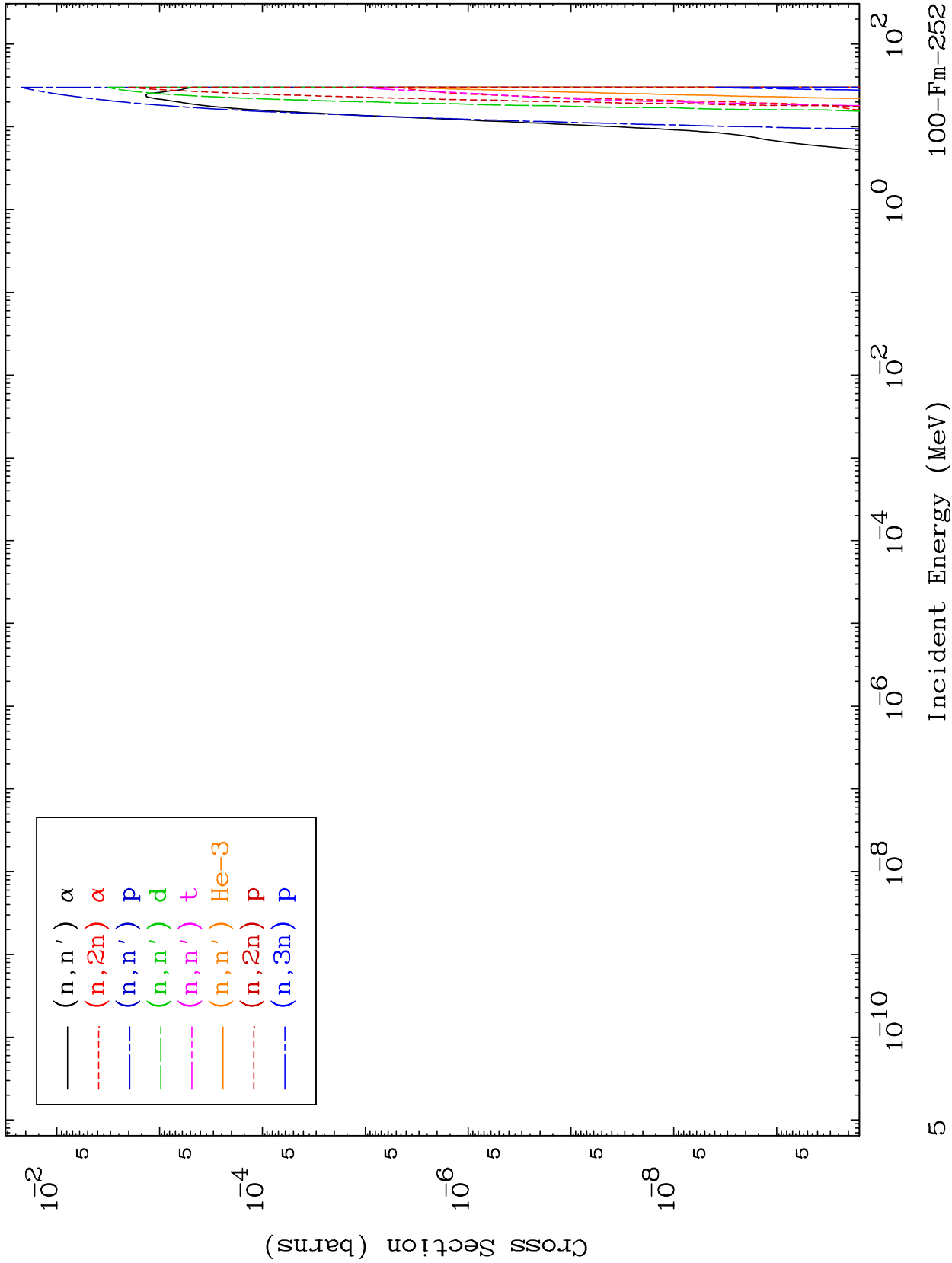
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MAT 9933

Charged Particle
293 Kelvin Cross Sections

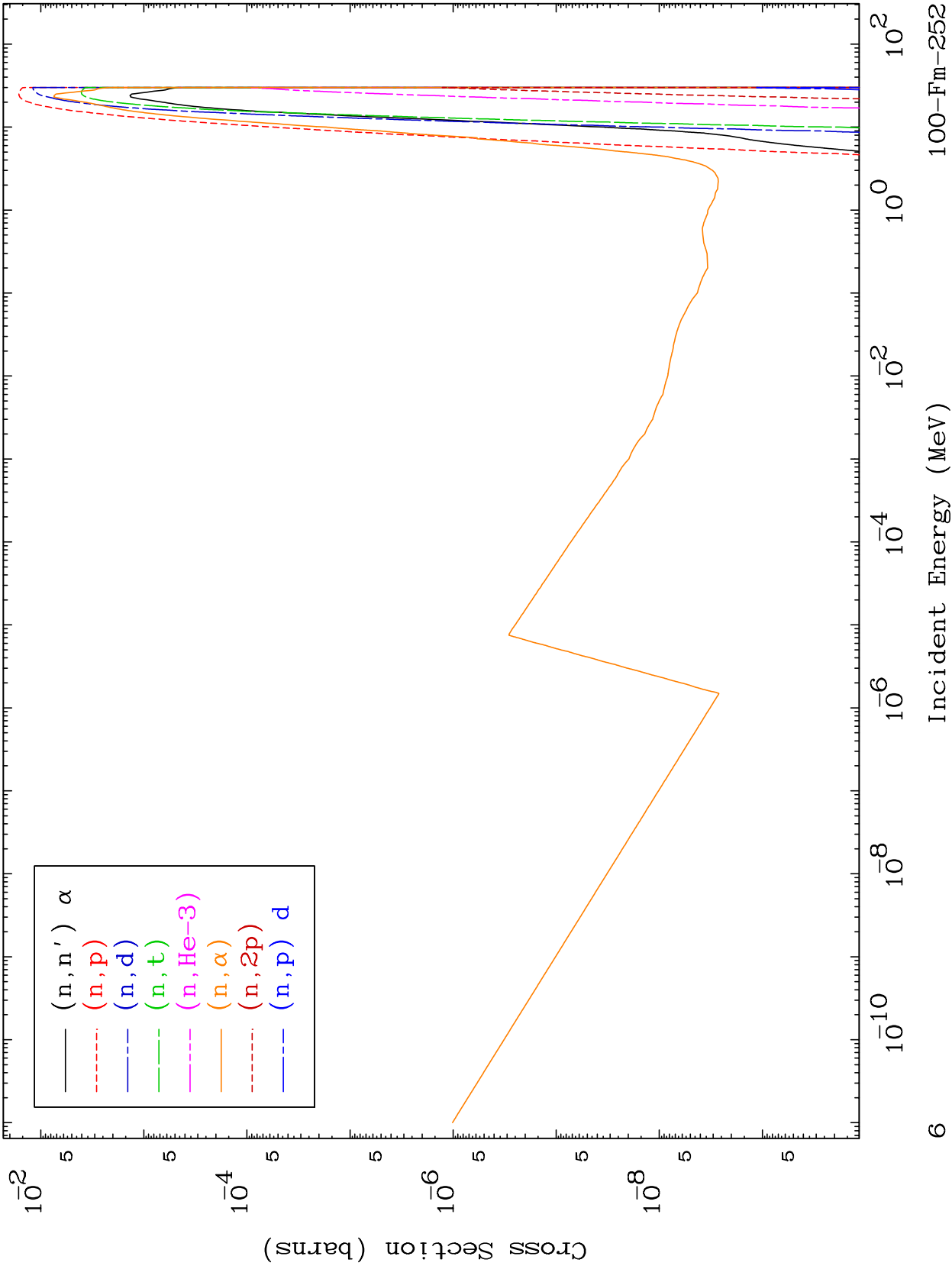
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MAT 9933

Charged Particle
293 Kelvin Cross Sections

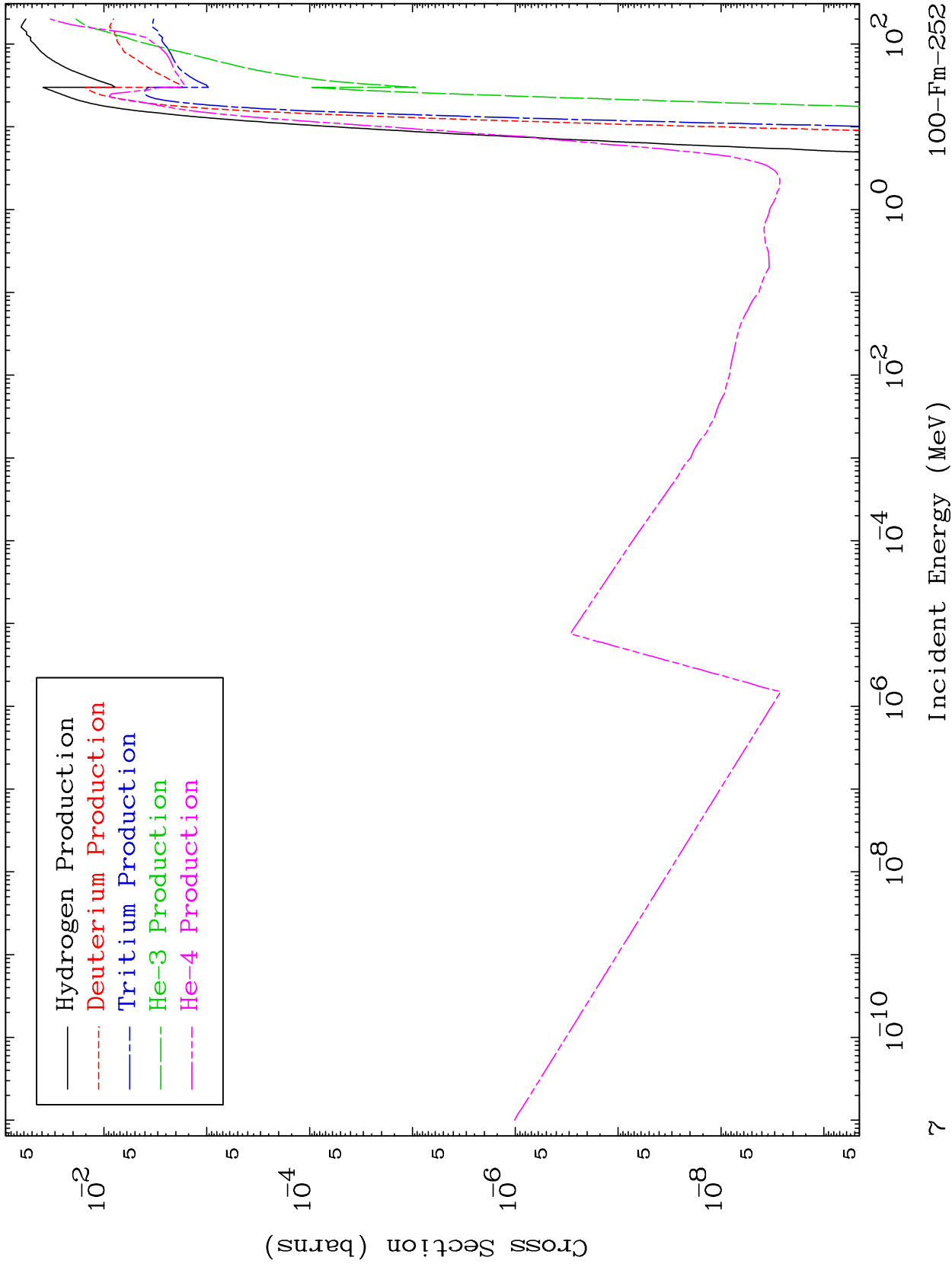
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MAT 9933

Particle Production
293 Kelvin Cross Sections

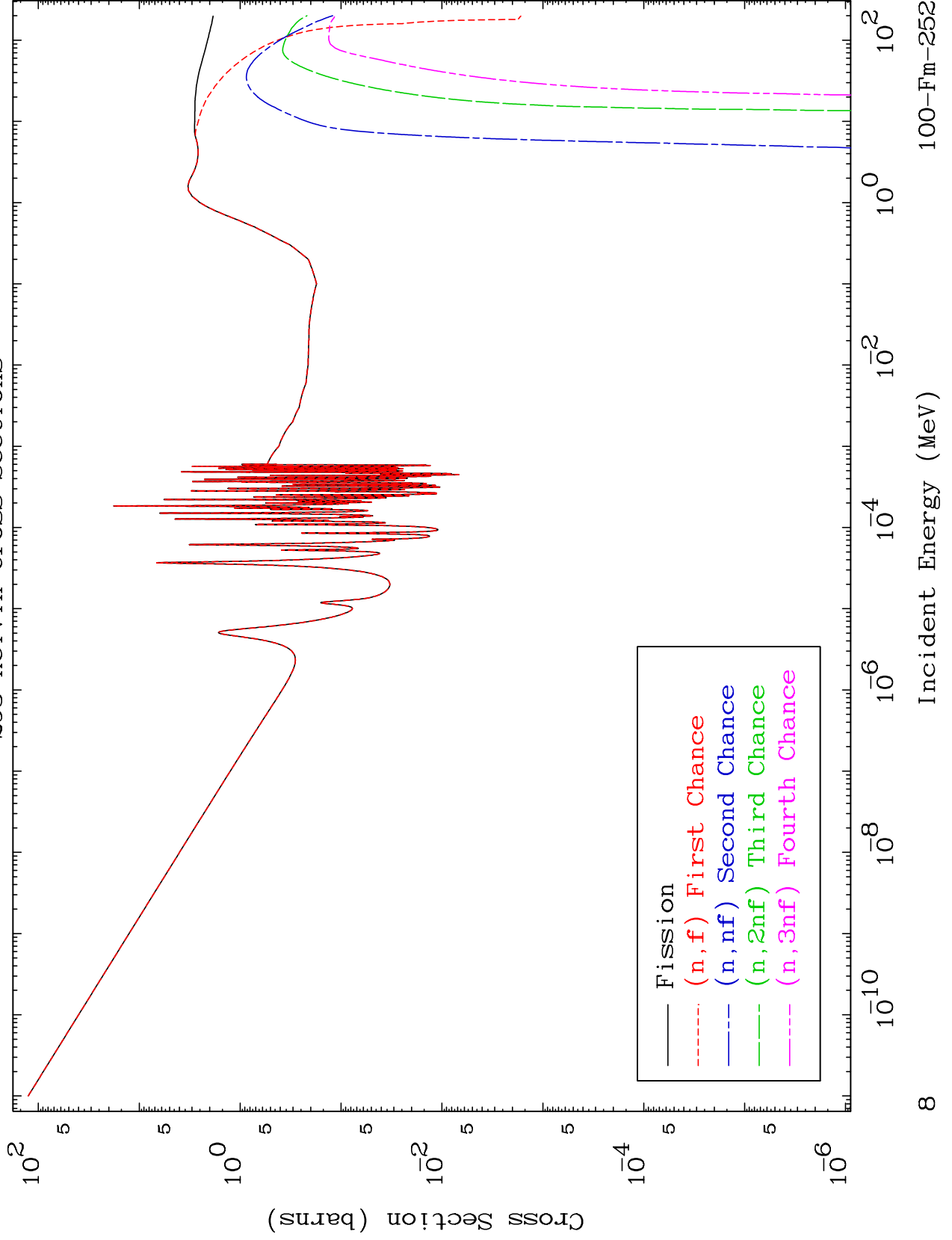
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MAT 9933

Fission
293 Kelvin Cross Sections

100-Fm-252

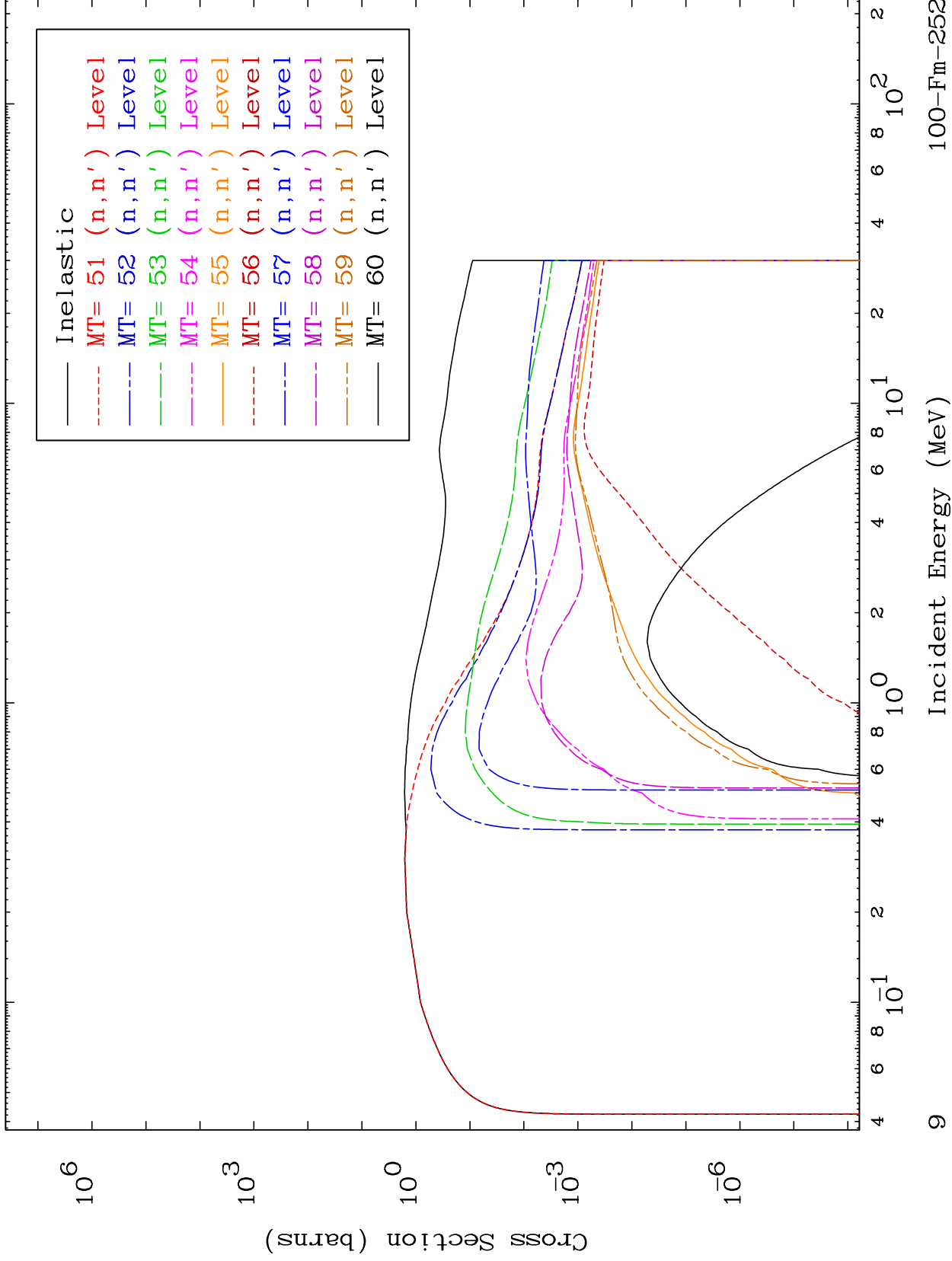


MAT 9933

(n,n') Levels

100-Fm-252

293 Kelvin Cross Sections

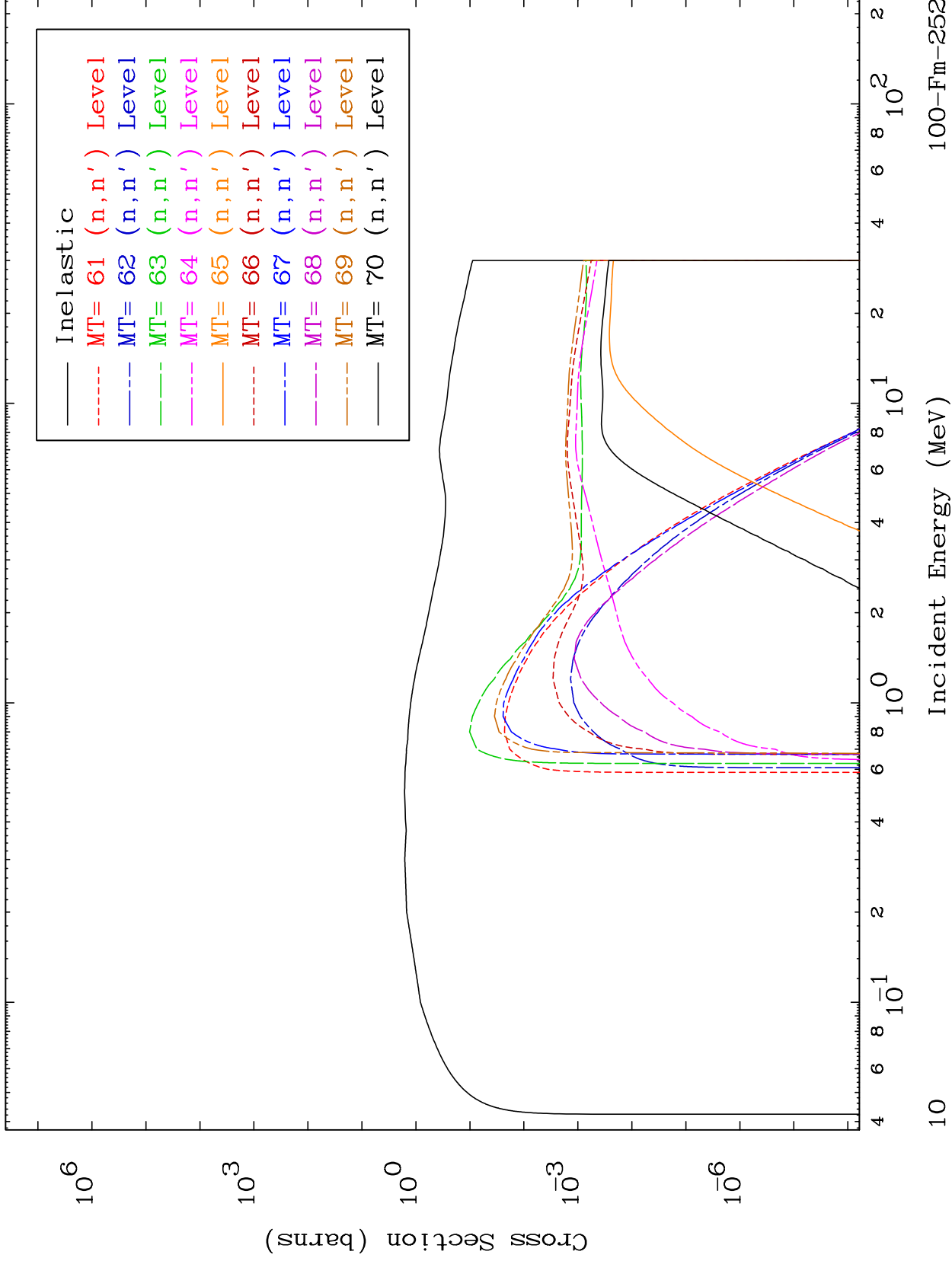


MAT 9933

(n,n') Levels

100-Fm-252

293 Kelvin Cross Sections



10

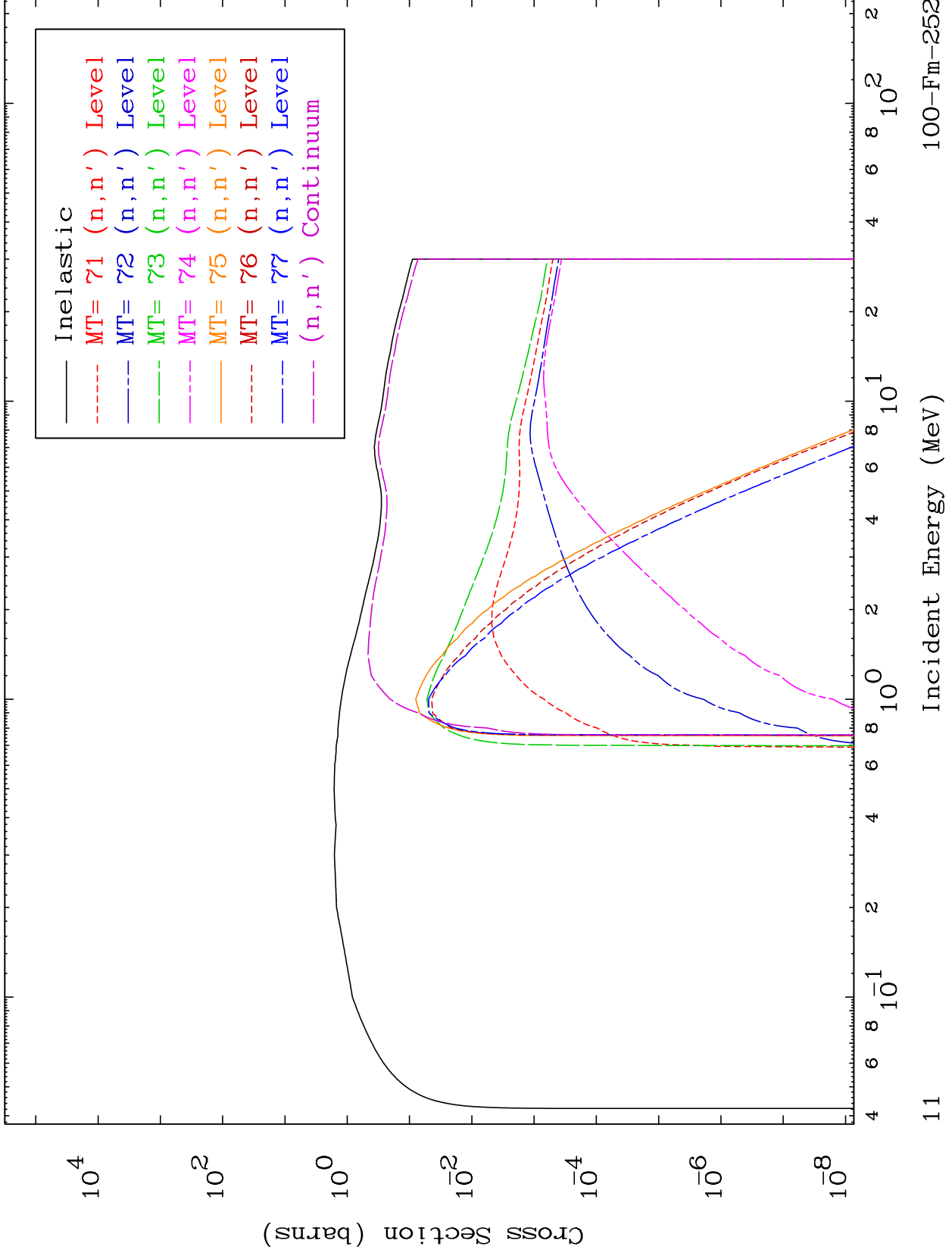
Incident Energy (MeV)

100-Fm-252

MAT 9933

(n,n') Levels
293 Kelvin Cross Sections

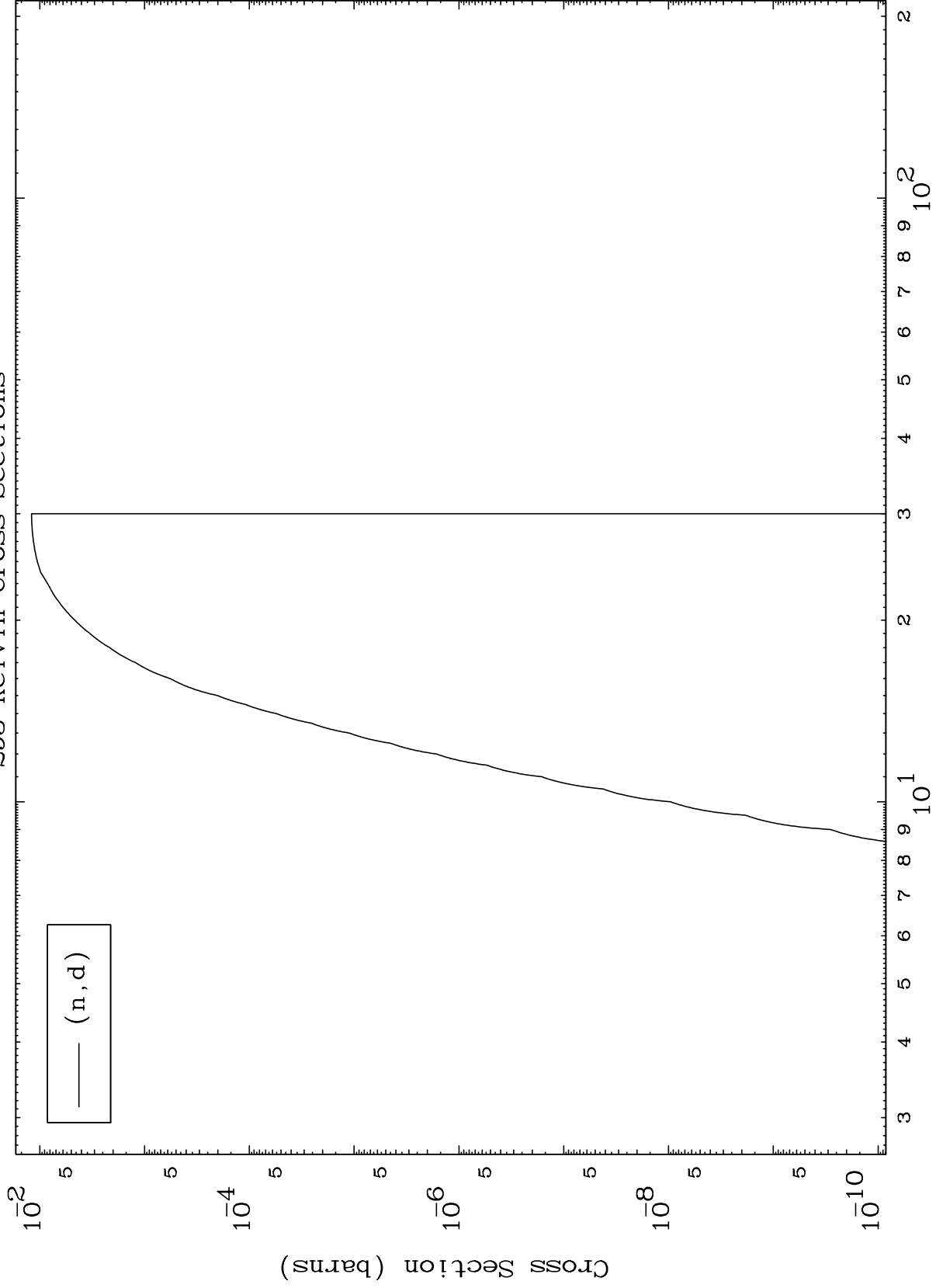
100-Fm-252



MAT 9933

(n,d) Levels
293 Kelvin Cross Sections

100-Fm-252



13

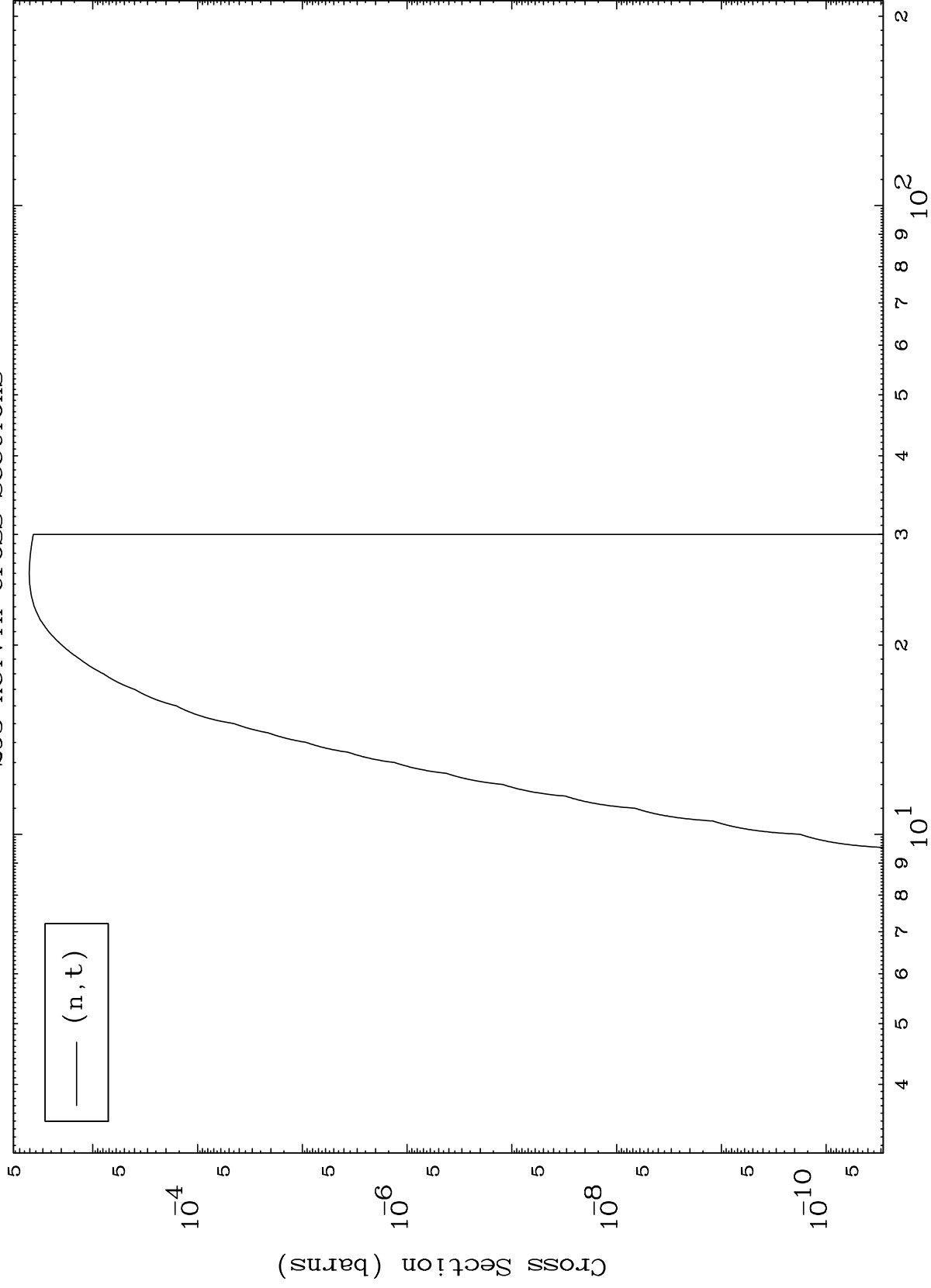
Incident Energy (MeV)

100-Fm-252

MAT 9933

(n,t) Levels
293 Kelvin Cross Sections

100-Fm-252



14

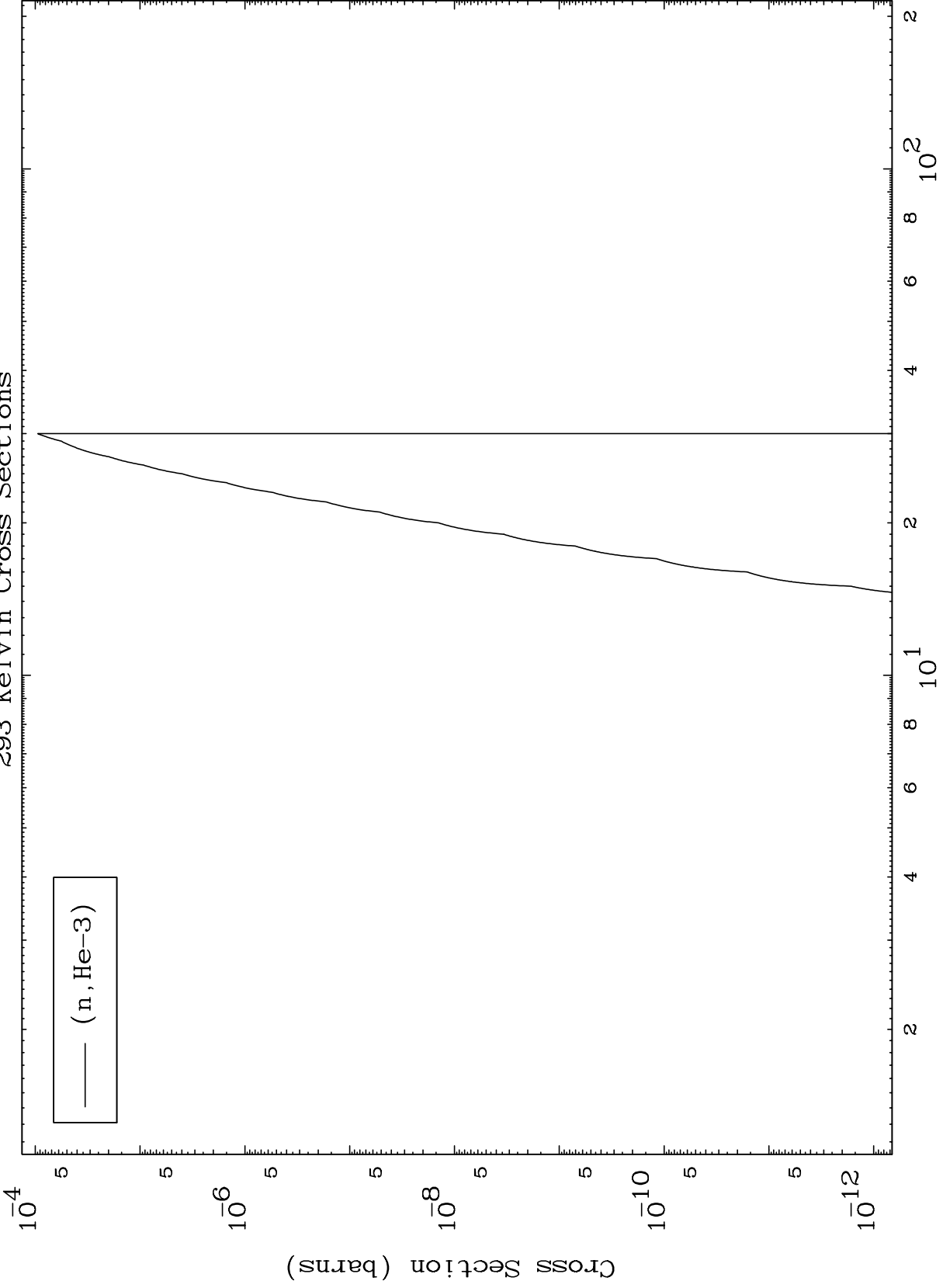
Incident Energy (MeV)

100-Fm-252

MAT 9933

(n,He3) Levels
293 Kelvin Cross Sections

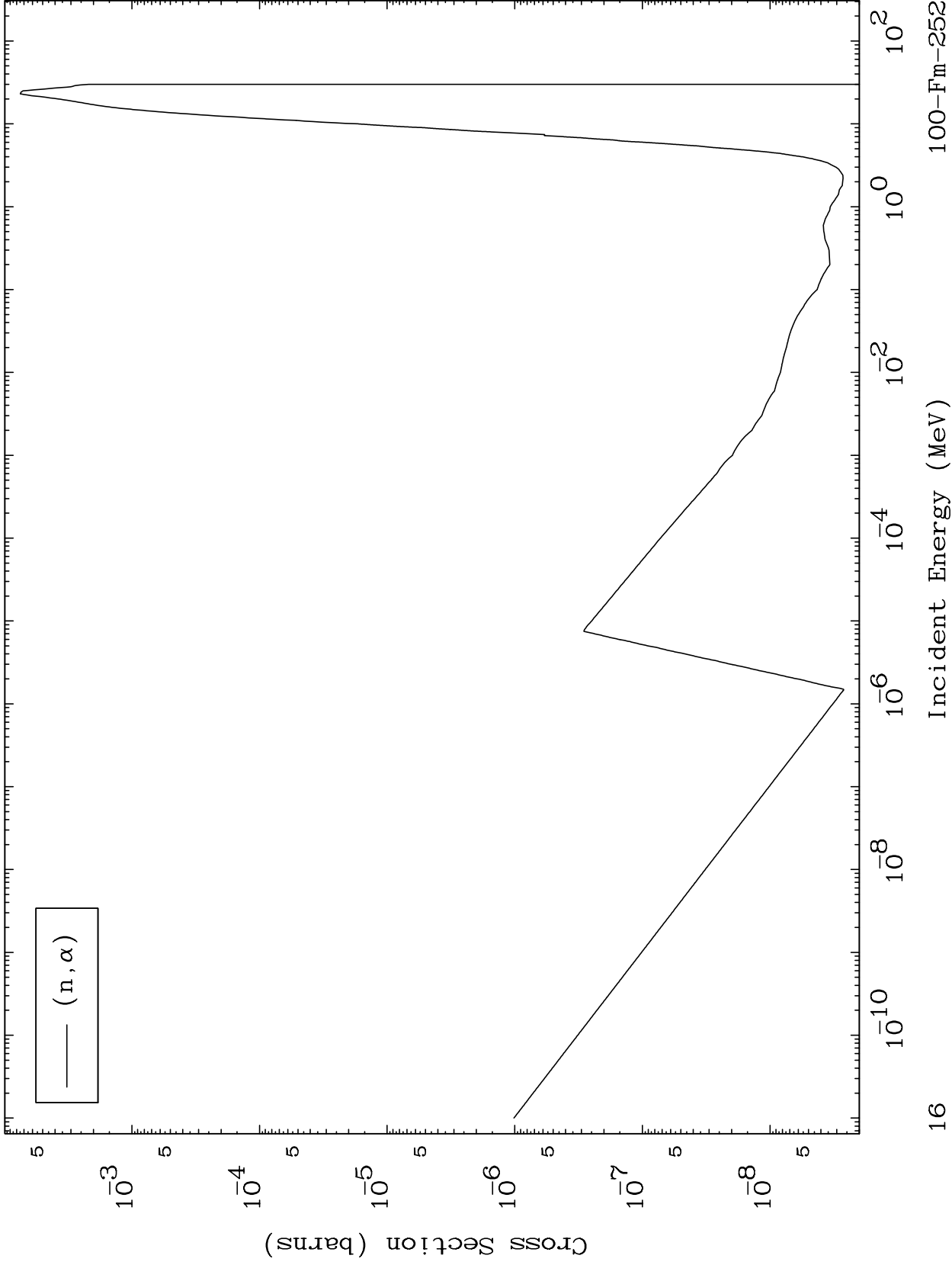
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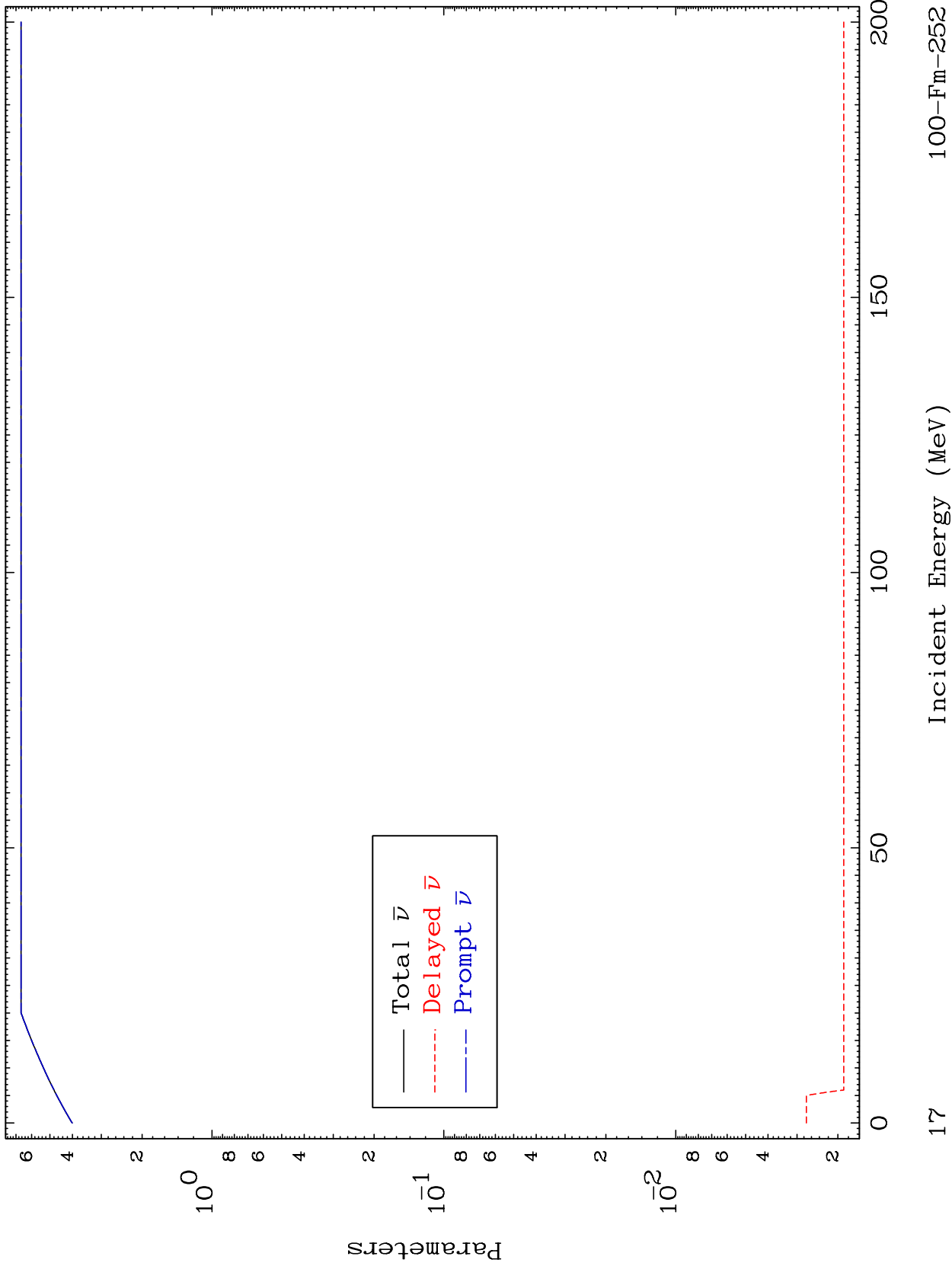


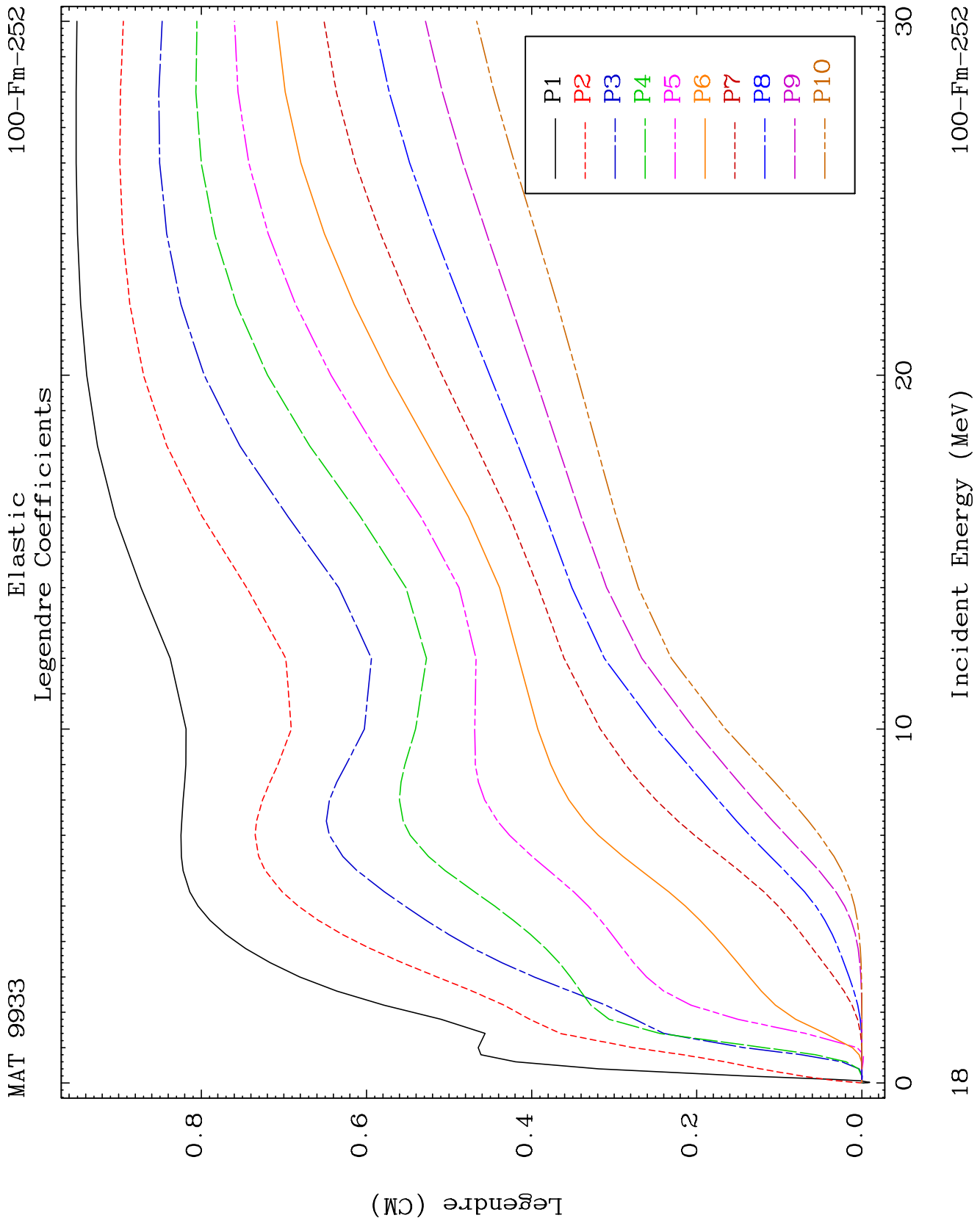
MAT 9933

(n, α) Levels
293 Kelvin Cross Sections

100-Fm-252



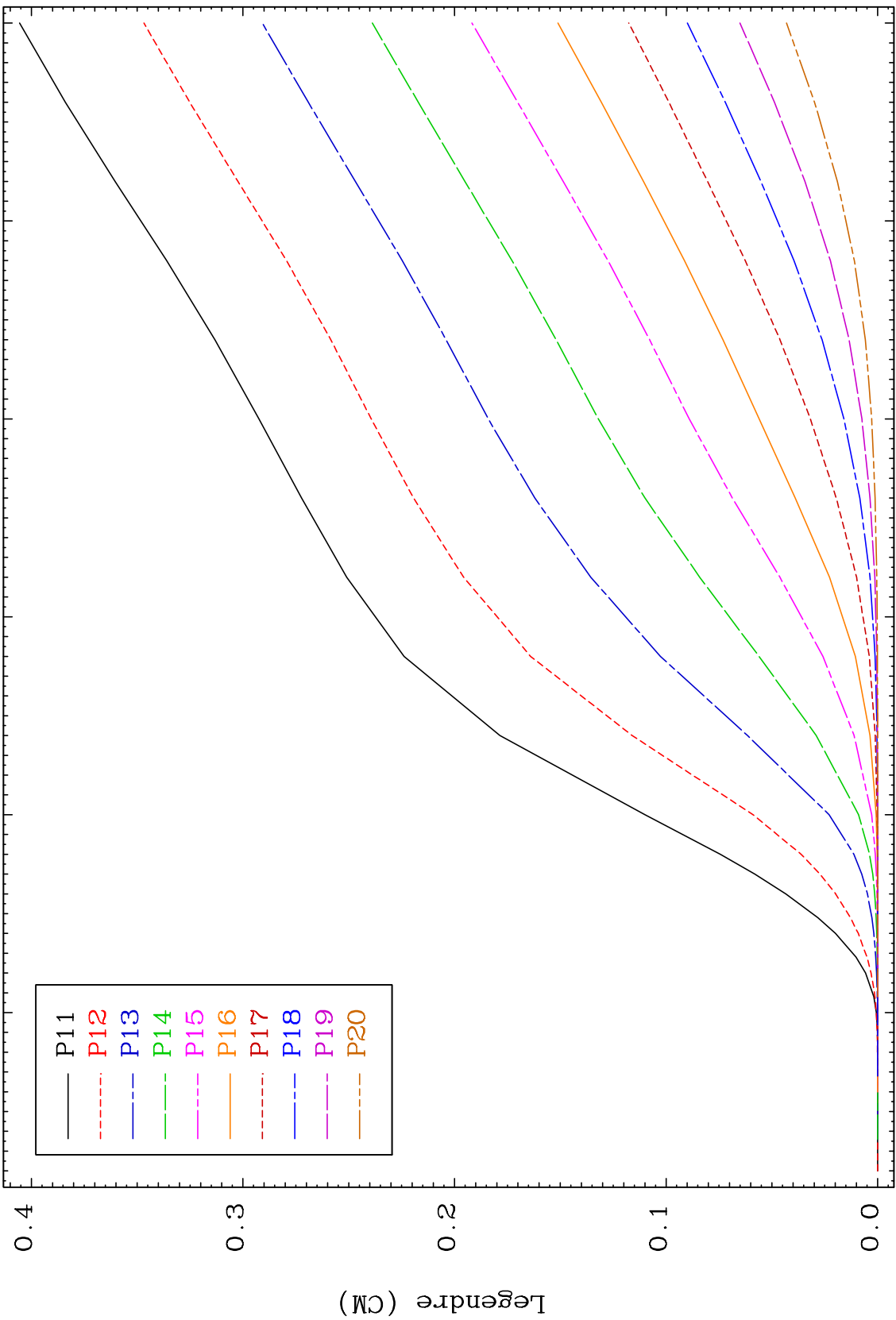




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Elastic Legendre Coefficients

100-Fm-252



19

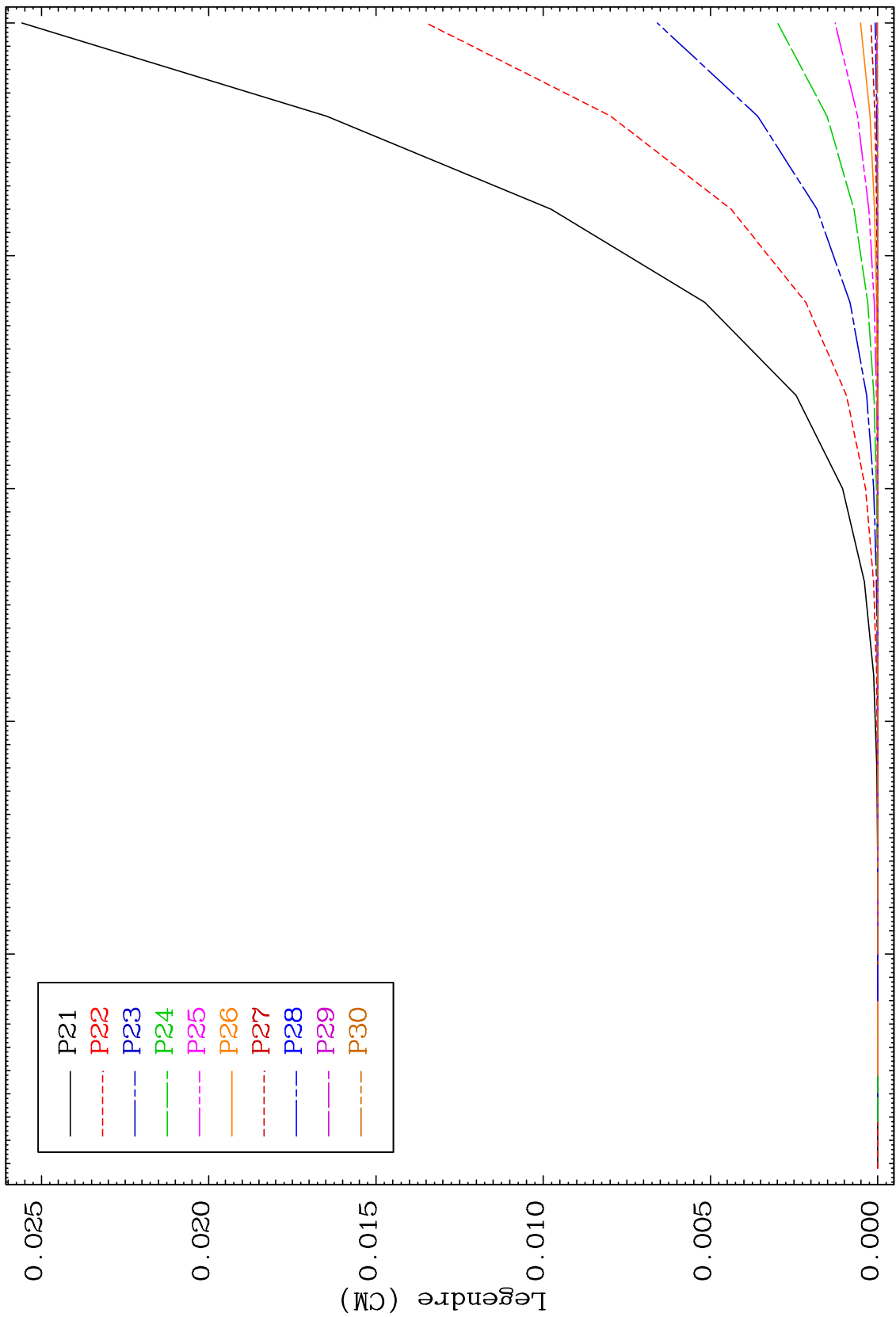
Incident Energy (MeV)

100-Fm-252

MAT 9933

Elastic
Legendre Coefficients

100-Fm-252



20

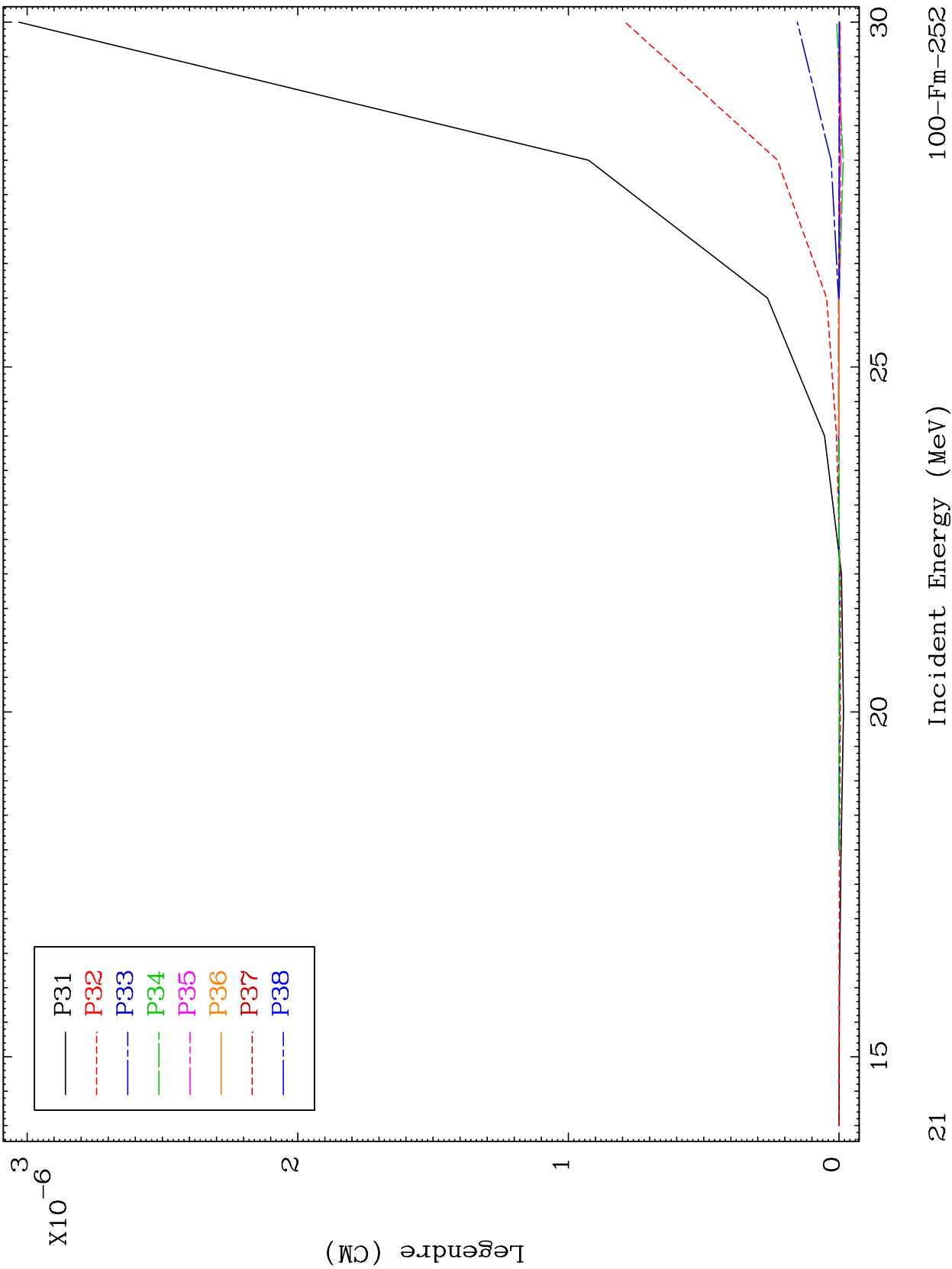
Incident Energy (MeV)

100-Fm-252

MAT 9933

Elastic
Legendre Coefficients

100-Fm-252



21

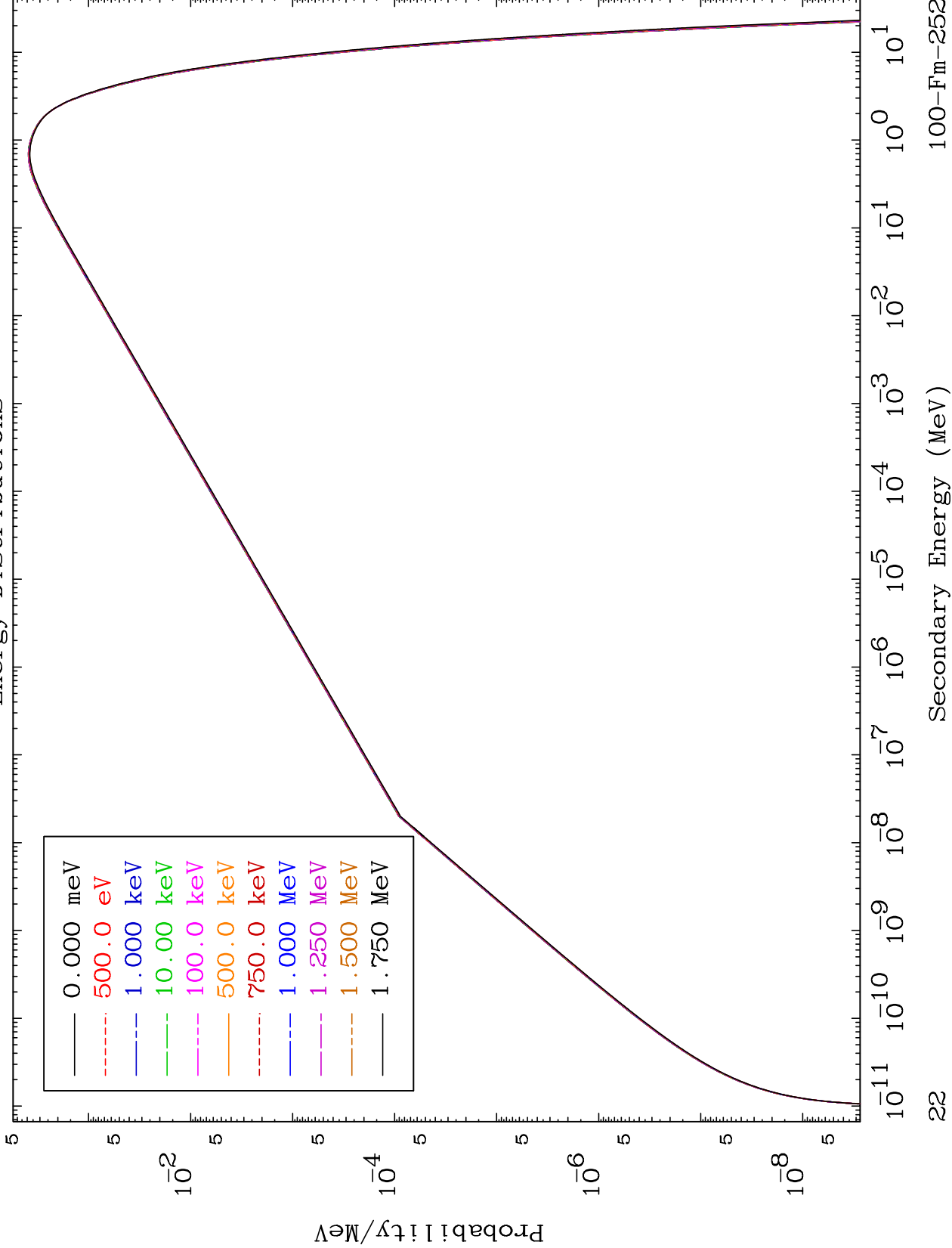
Incident Energy (MeV)

100-Fm-252

MAT 9933

Fission Energy Distributions

100-Fm-252



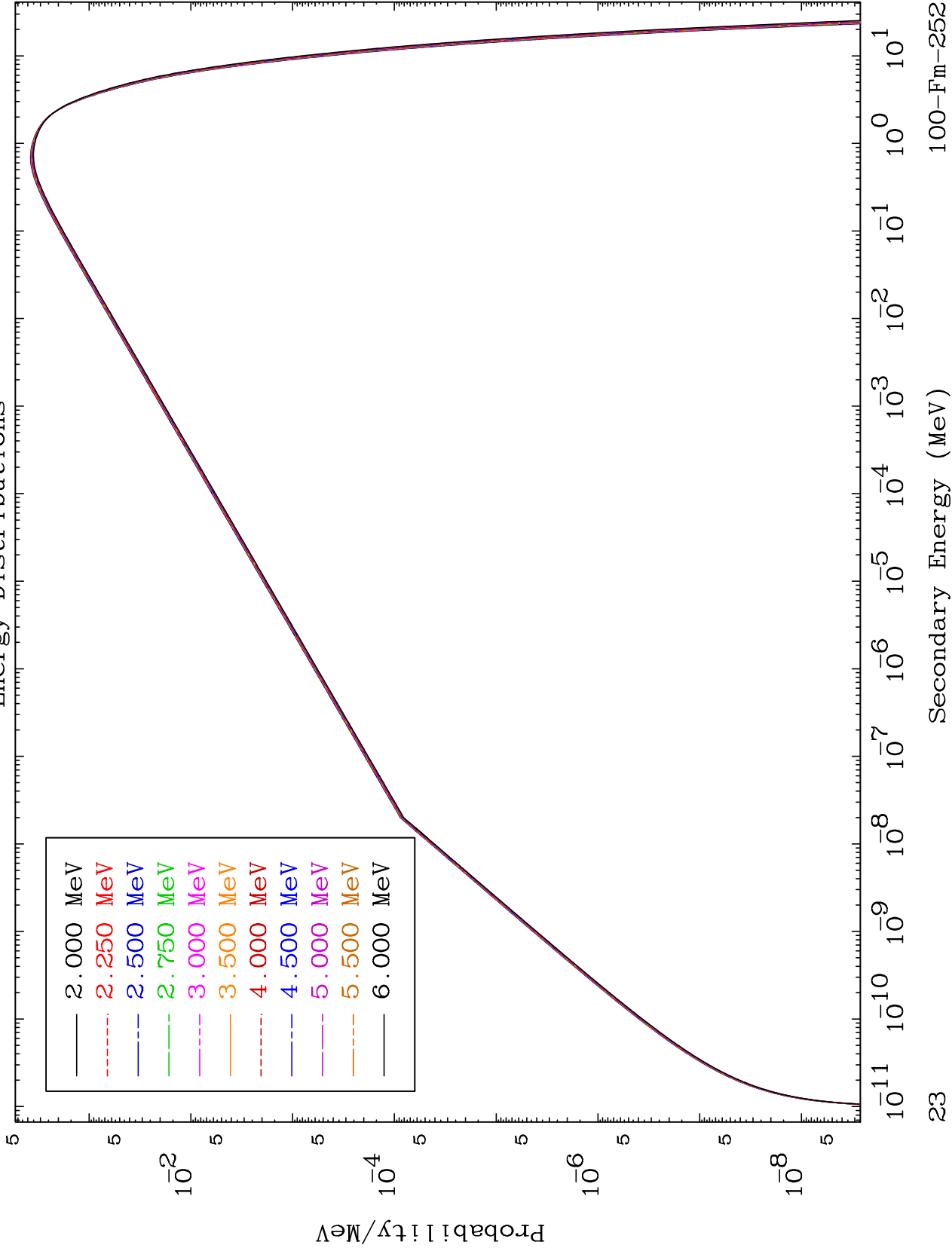
100-Fm-252

Secondary Energy (MeV)

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Fission Energy Distributions

100-Fm-252

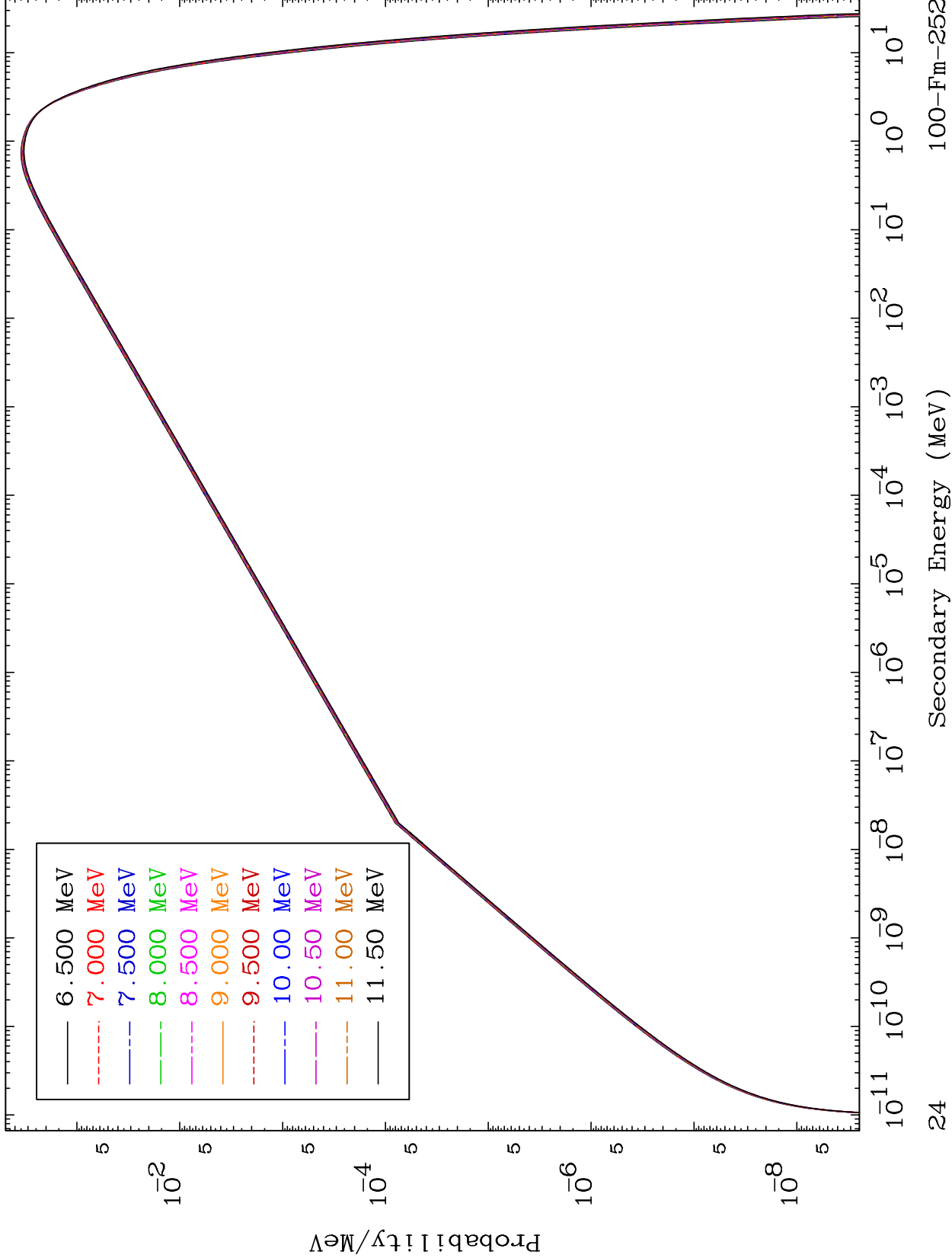


100-Fm-252

MAT 9933

Fission Energy Distributions

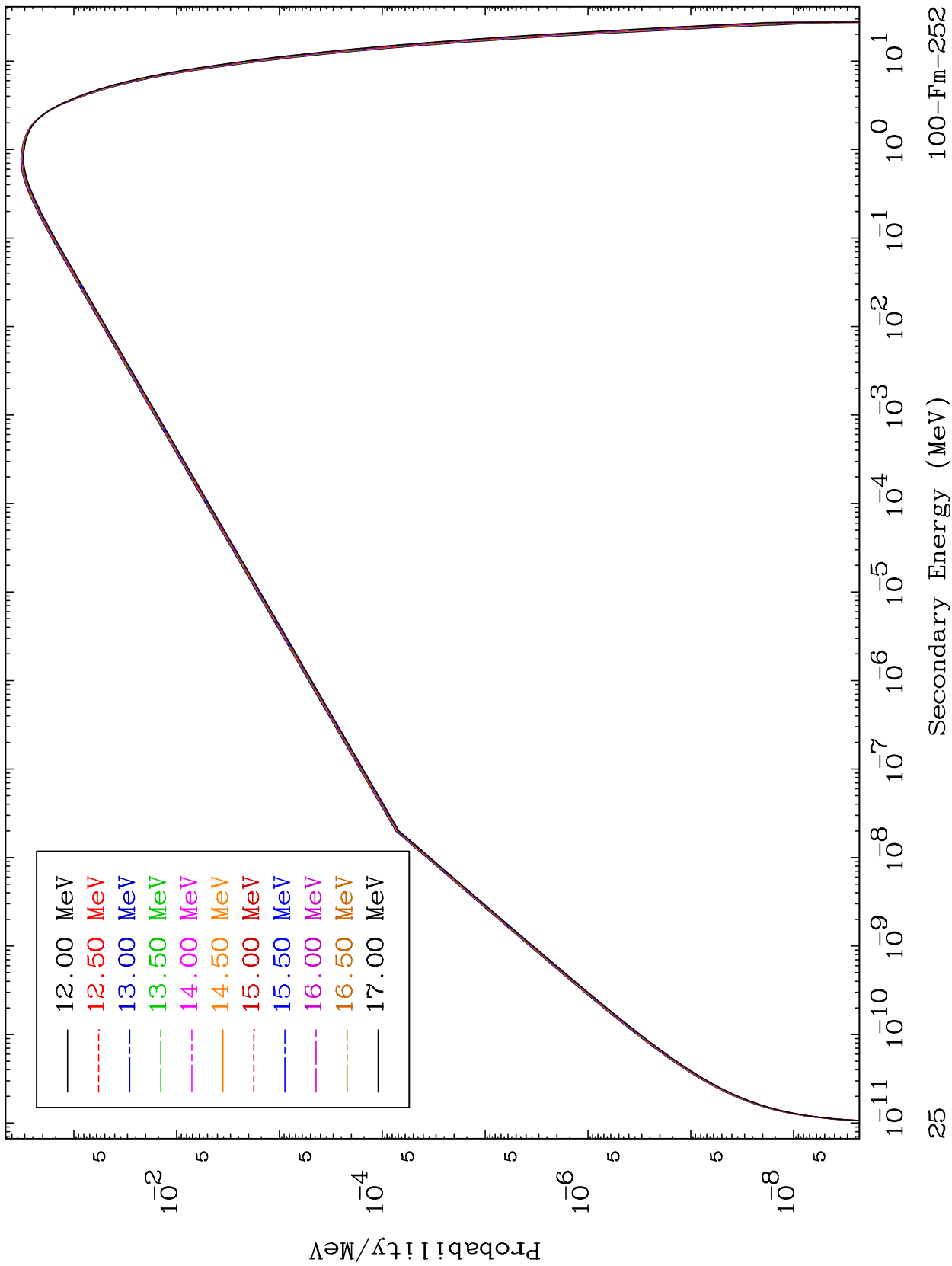
100-Fm-252



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Fission Energy Distributions

100-Fm-252



100-Fm-252

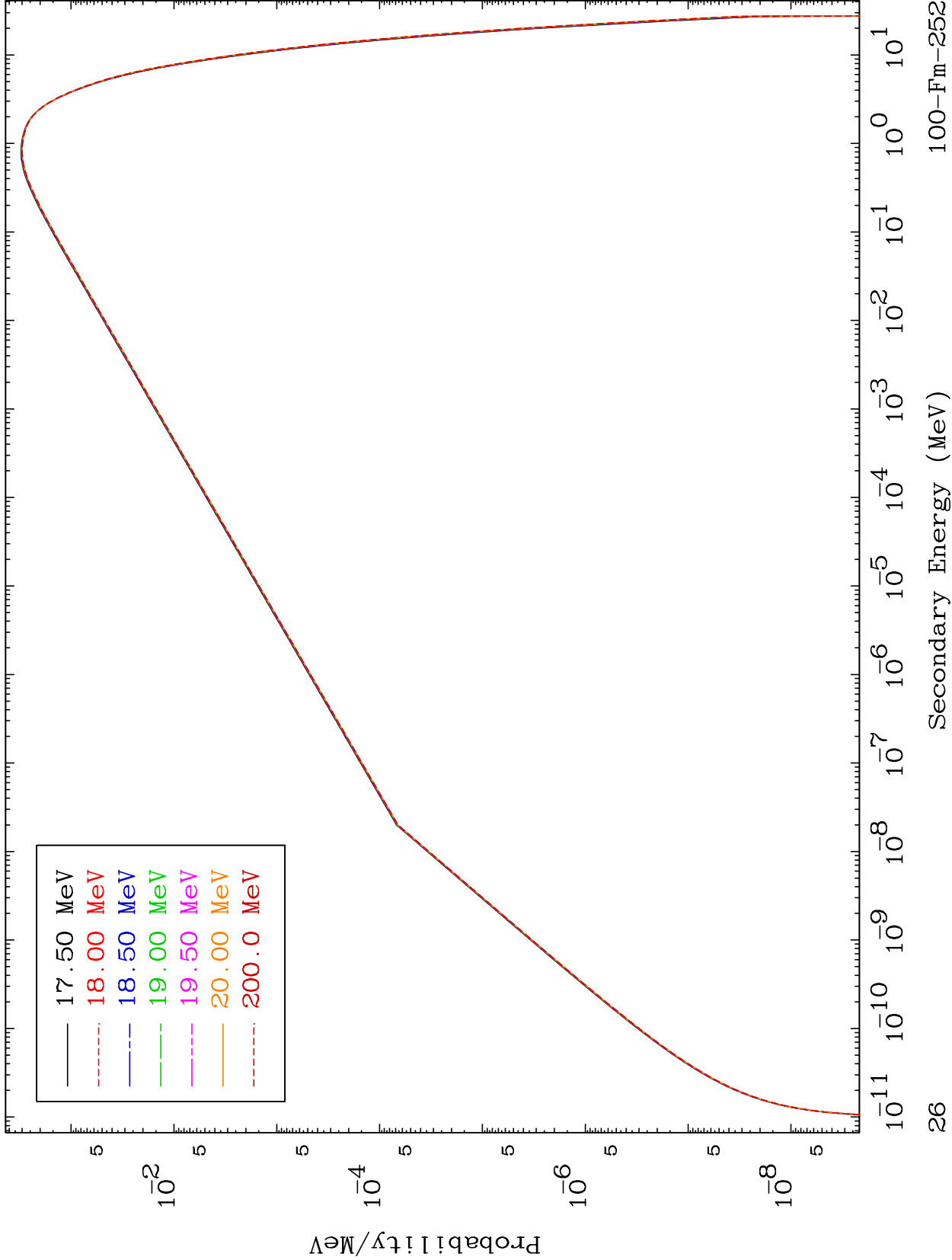
Secondary Energy (MeV)

25

MAT 9933

Fission Energy Distributions

100-Fm-252

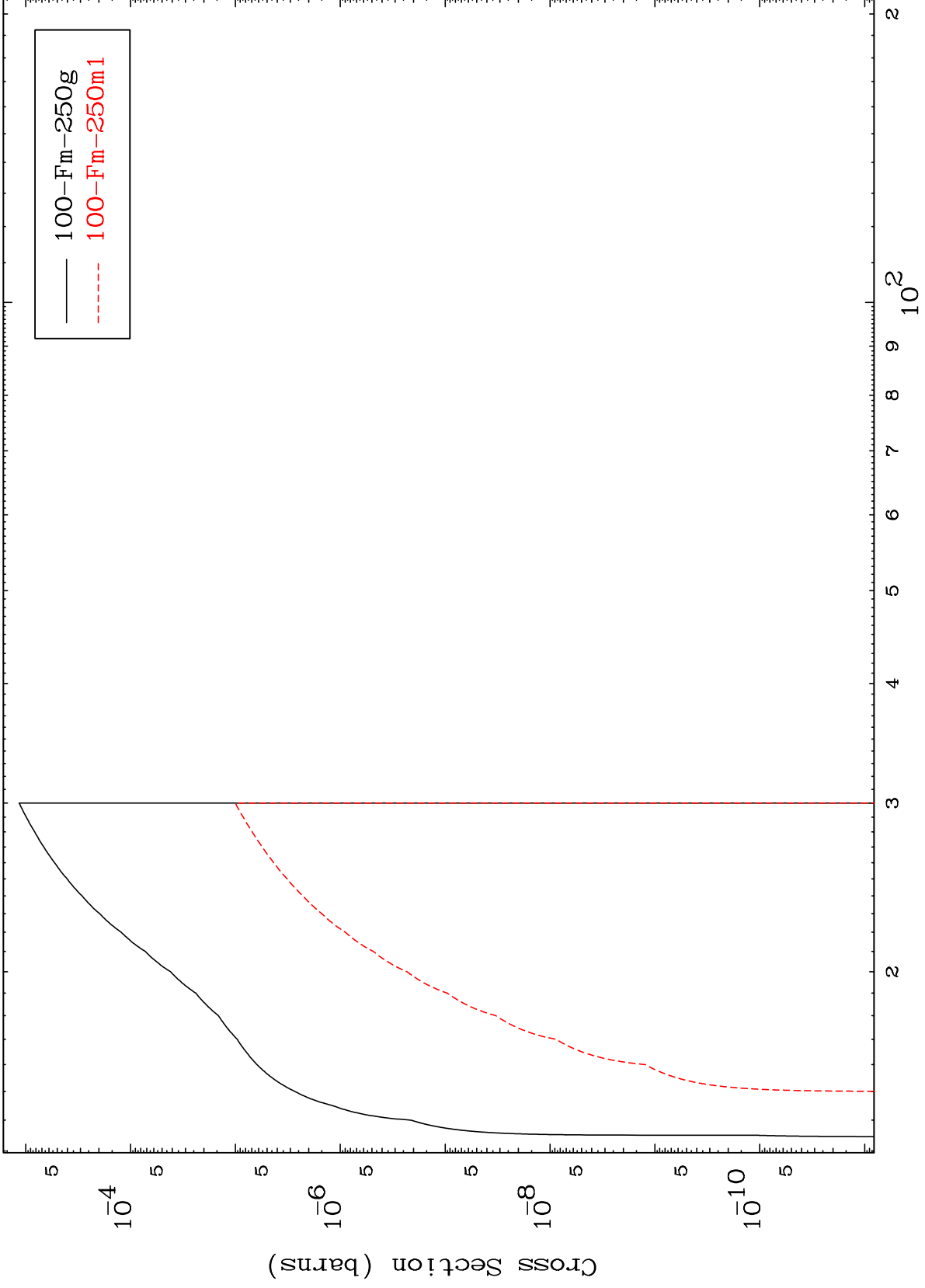


MAT 9933

(n,3n)

100-Fm-252

Radionuclide Production Cross Section



27

Incident Energy (MeV)

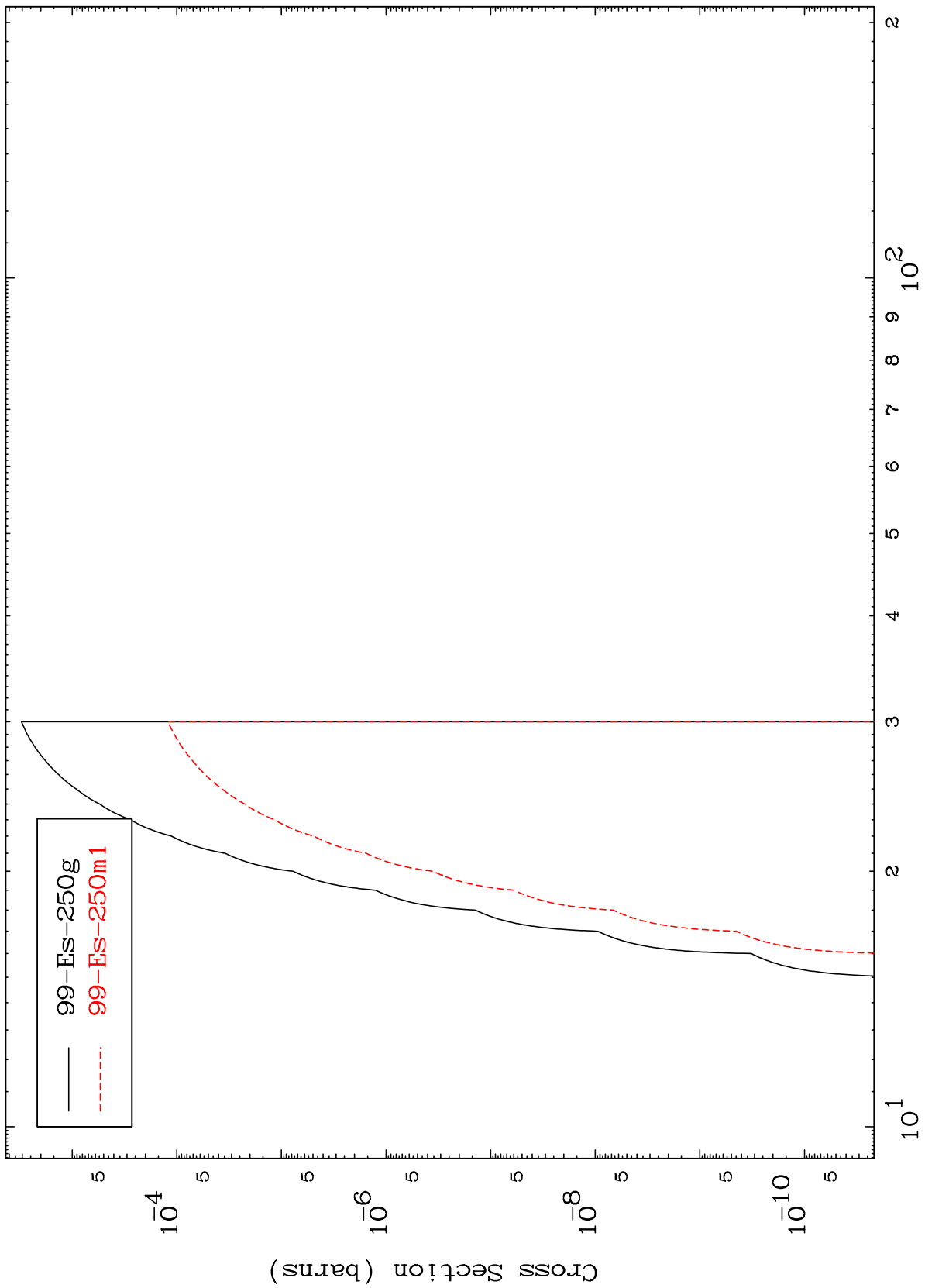
100-Fm-252

MAT 9933

(n,n') d

100-Fm-252

Radionuclide Production Cross Section



28

Incident Energy (MeV)

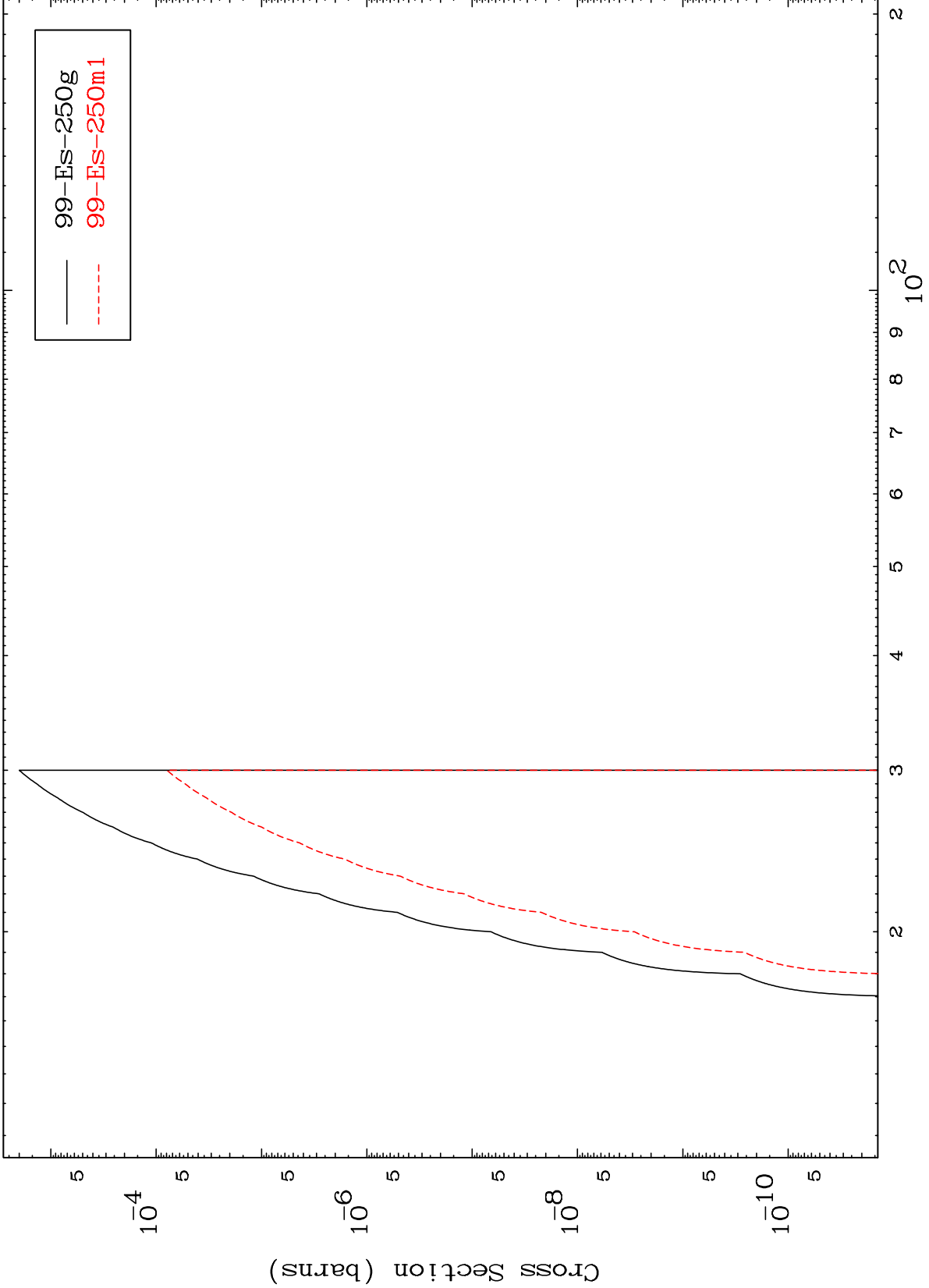
100-Fm-252

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(n,2n) p

100-Fm-252

Radionuclide Production Cross Section



29

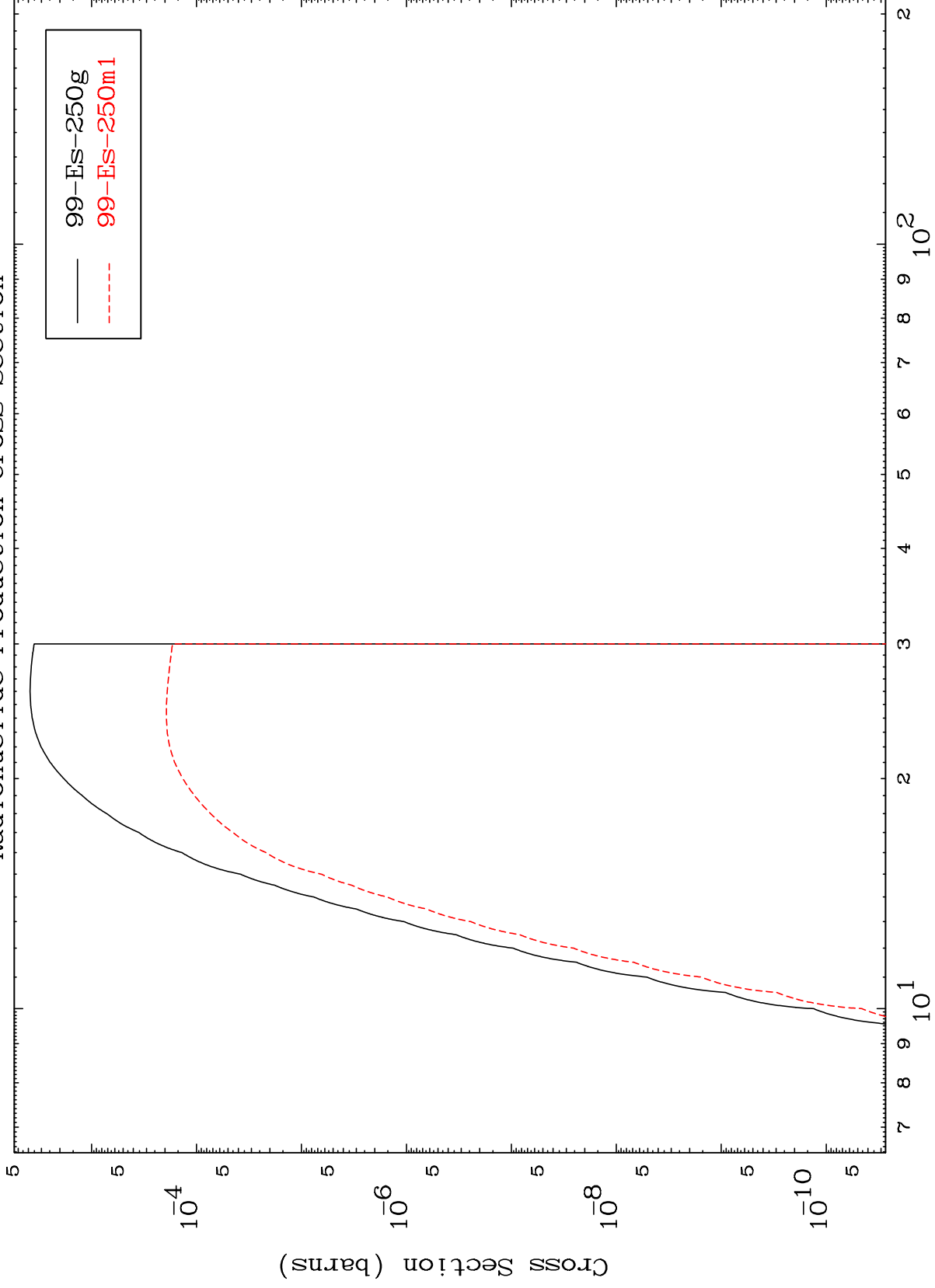
Incident Energy (MeV)

100-Fm-252

MAT 9933

100-Fm-252

(n, t)
Radionuclide Production Cross Section



100-Fm-252

Incident Energy (MeV)

30