

Program EVALPLOT
(Version 2021-1)

by

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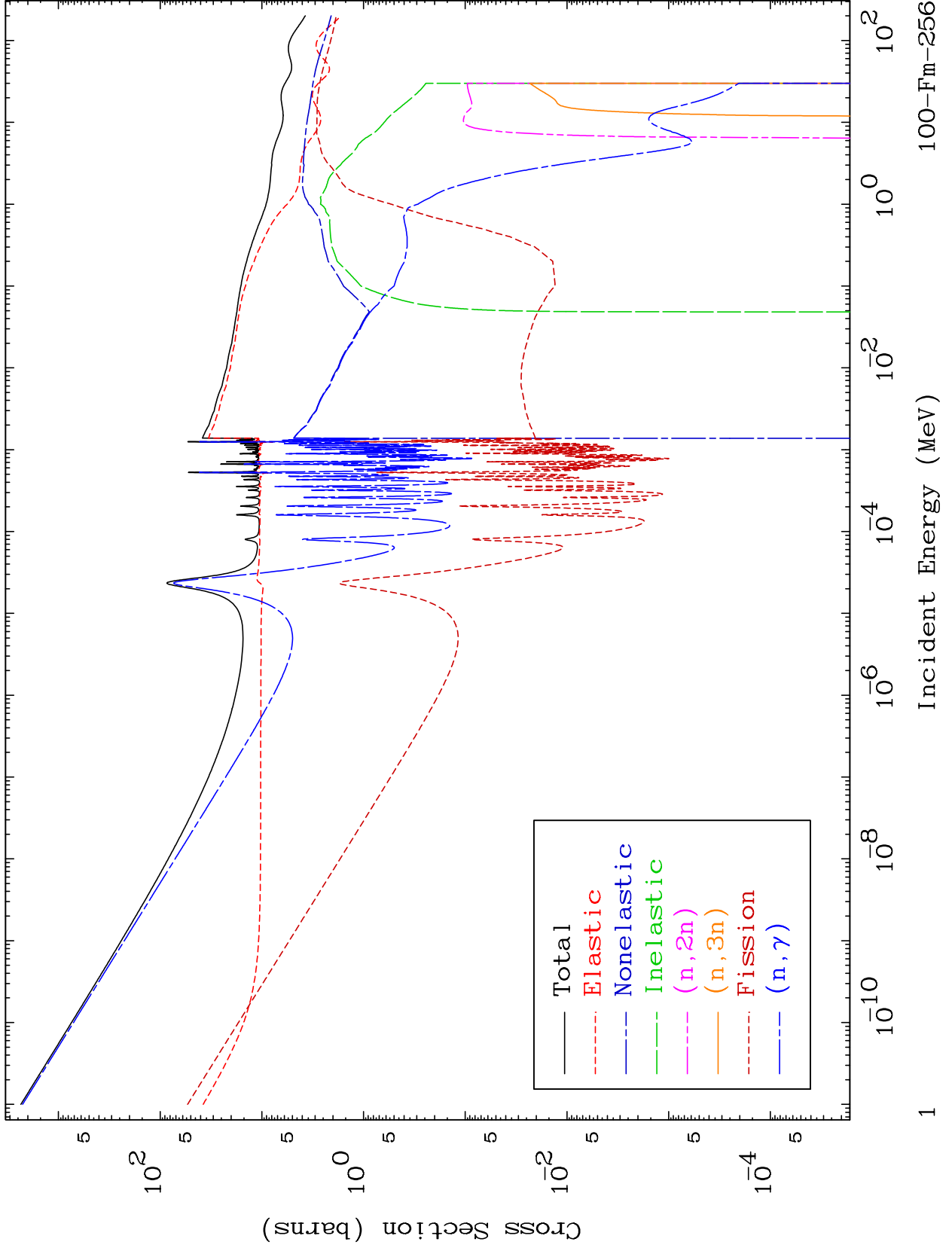
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9937

Neutron Major
293 Kelvin Cross Sections

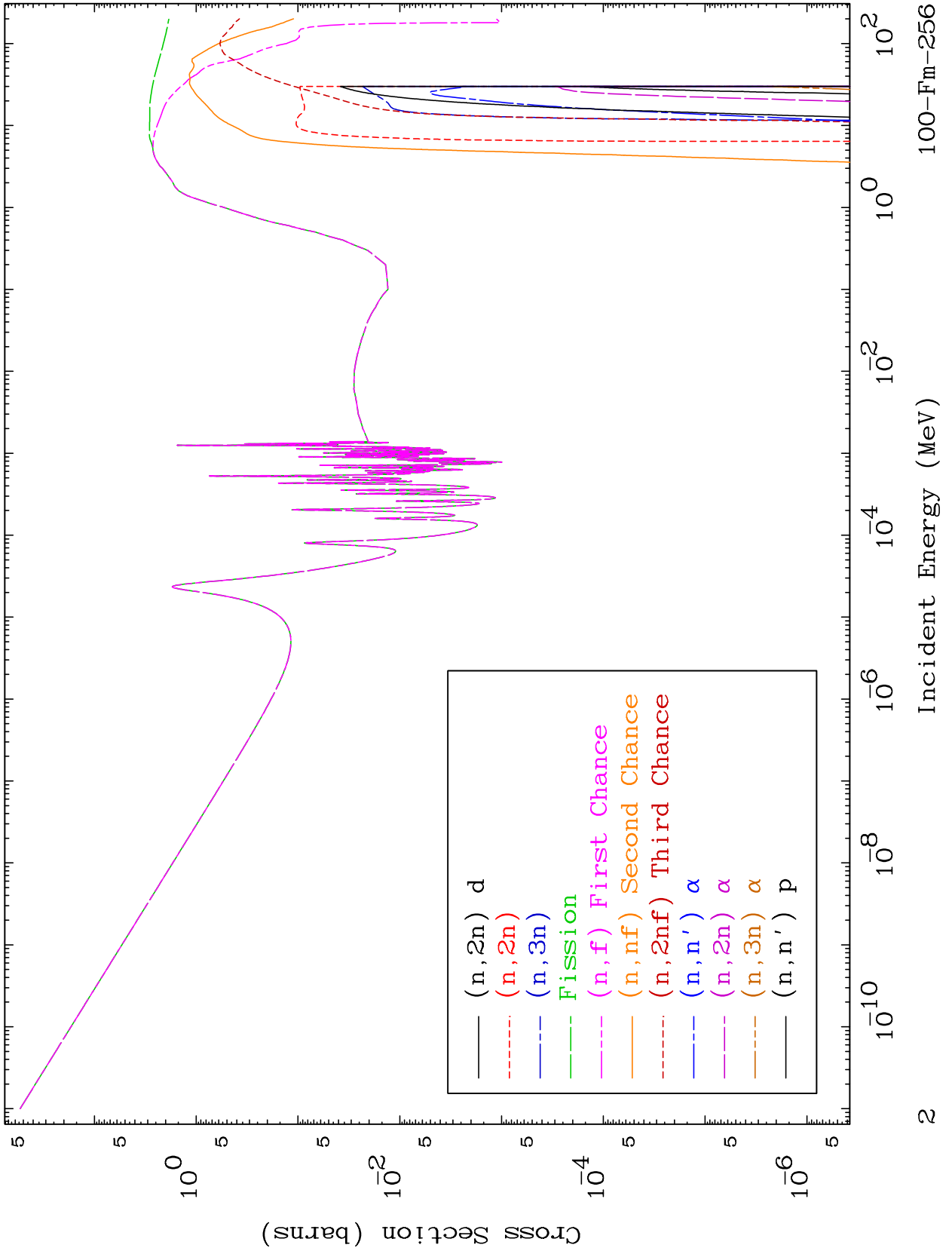
100-Fm-256



MAT 9937

Neutron Absorption
293 Kelvin Cross Sections

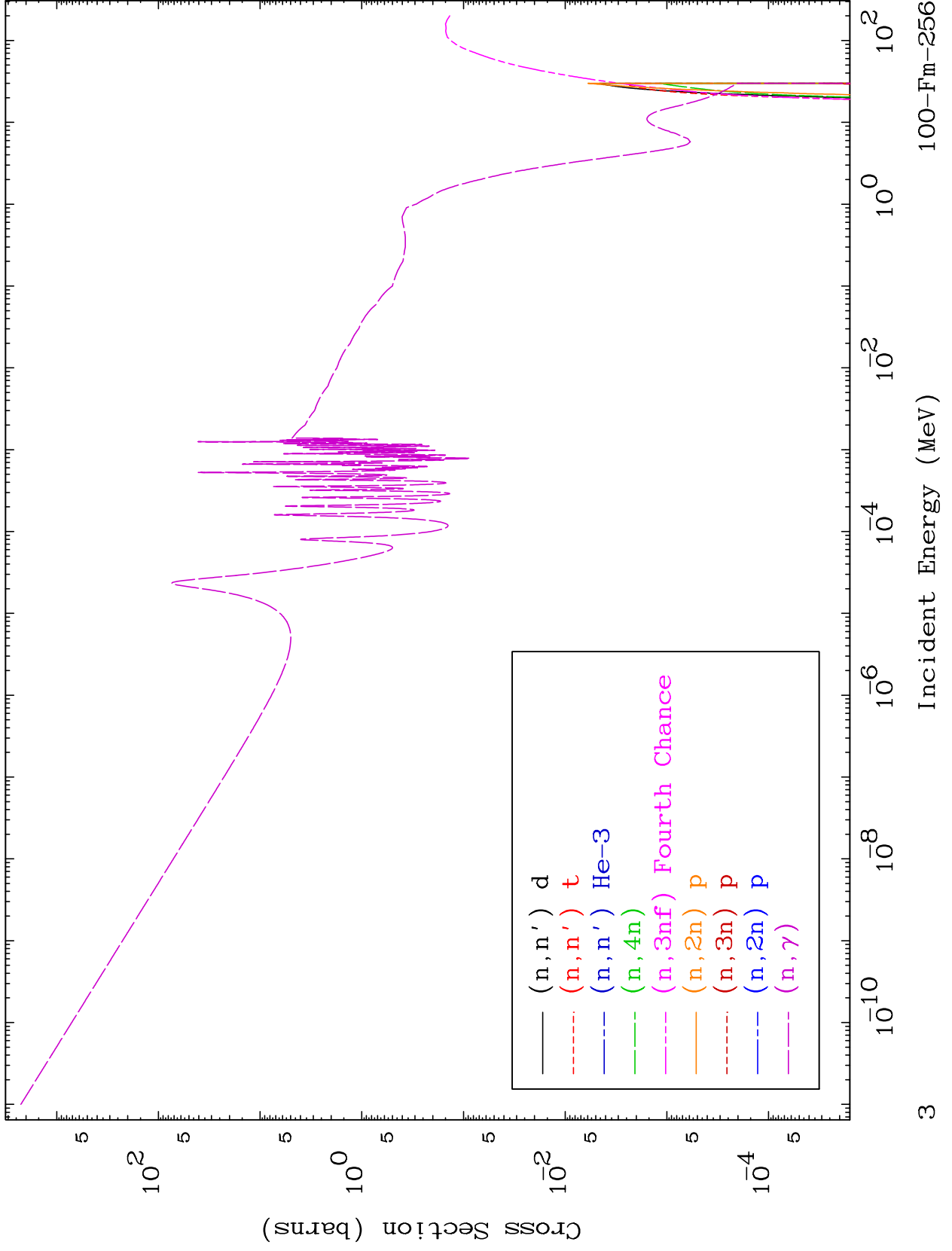
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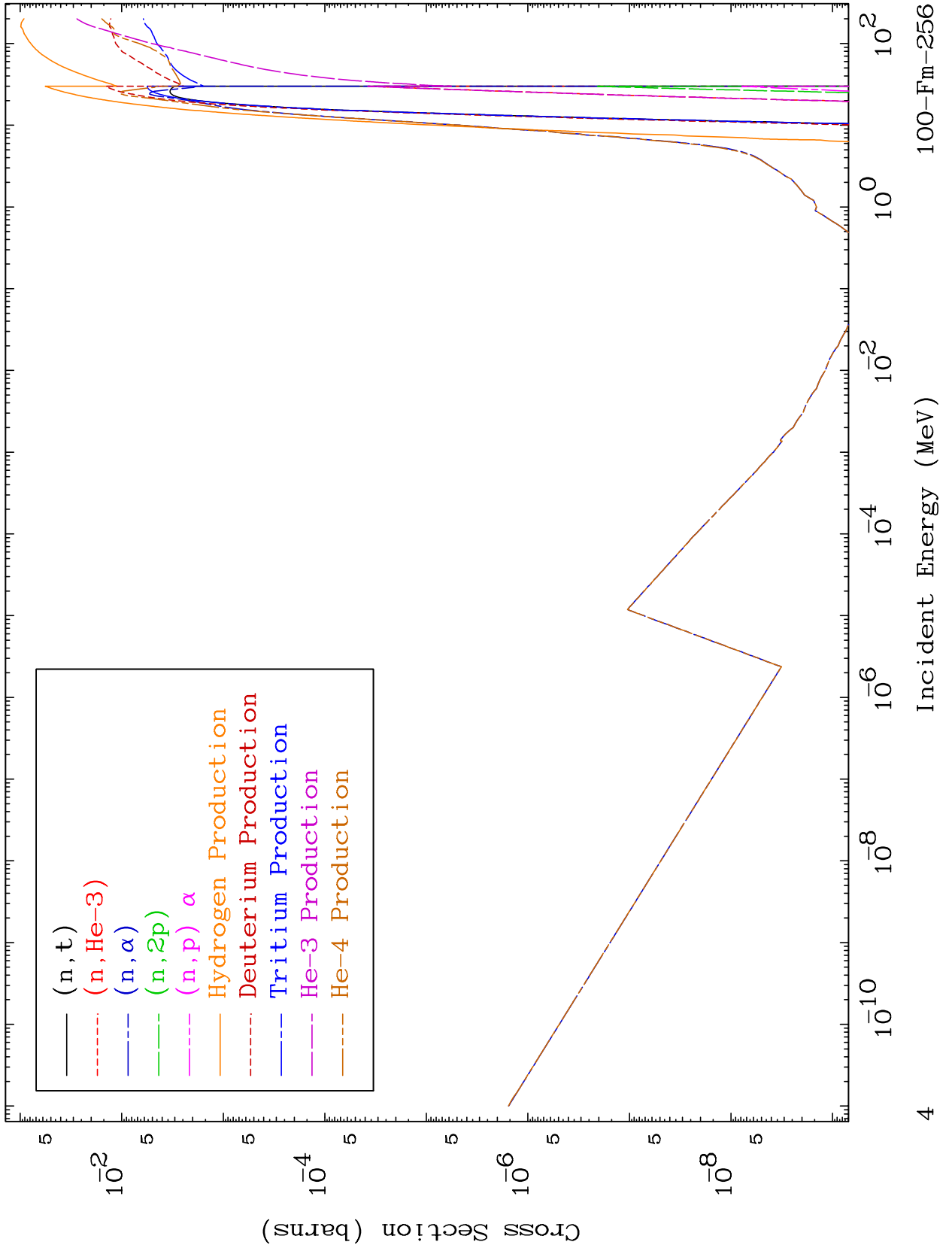


MAT 9937

Neutron Absorption
293 Kelvin Cross Sections

100-Fm-256

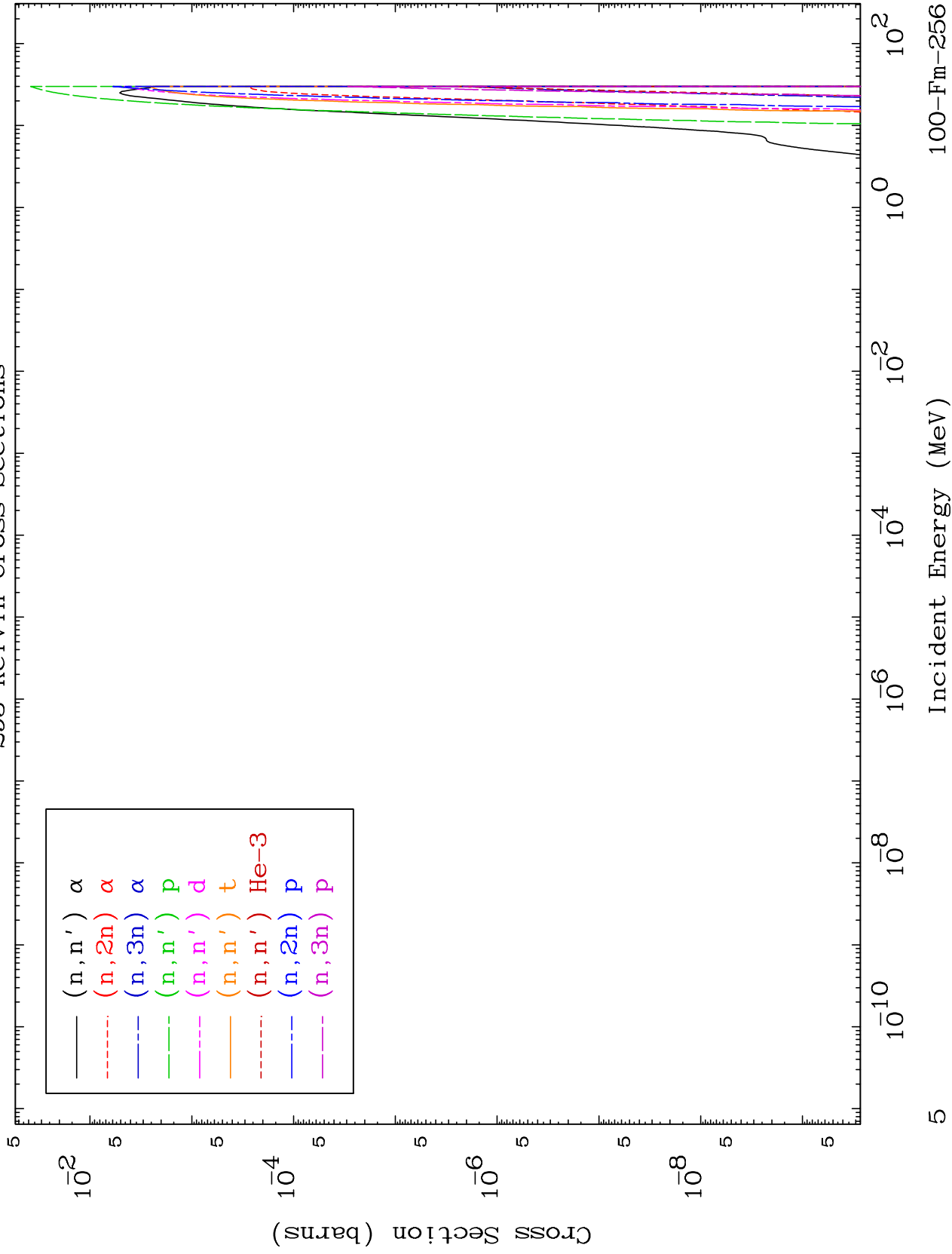




MAT 9937

Charged Particle
293 Kelvin Cross Sections

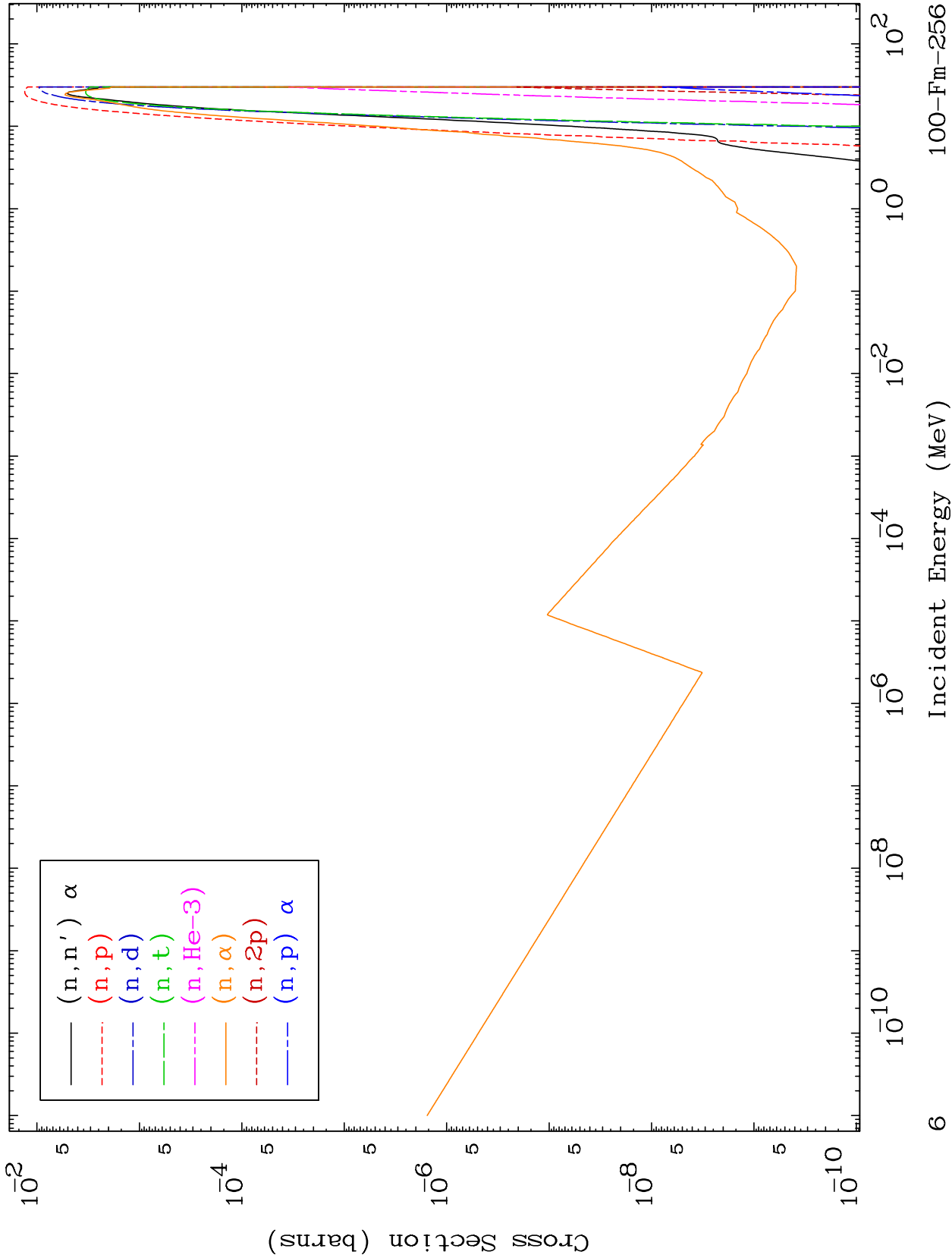
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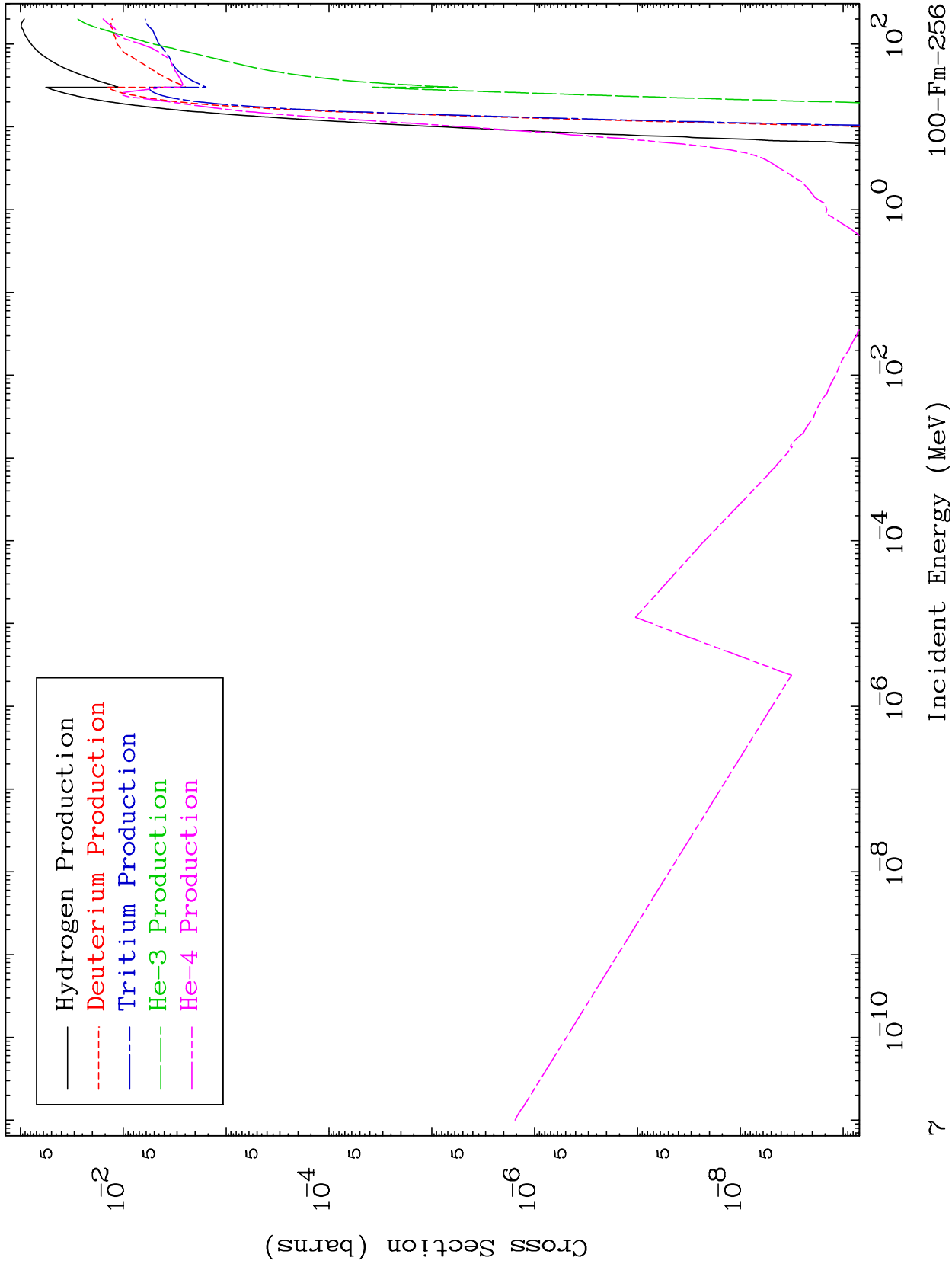


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Charged Particle
293 Kelvin Cross Sections

100-Fm-256

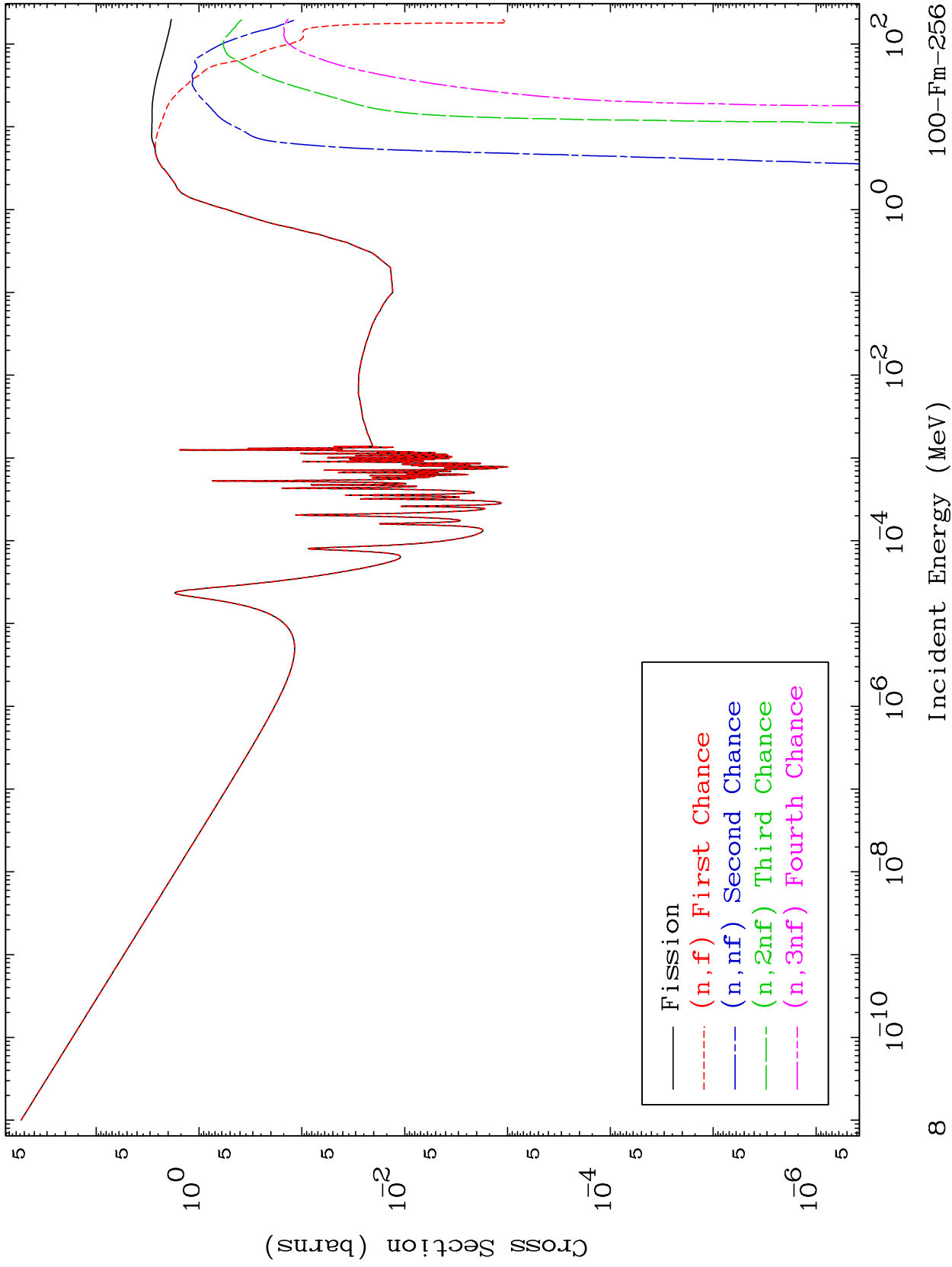




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Fission
293 Kelvin Cross Sections

100-Fm-256

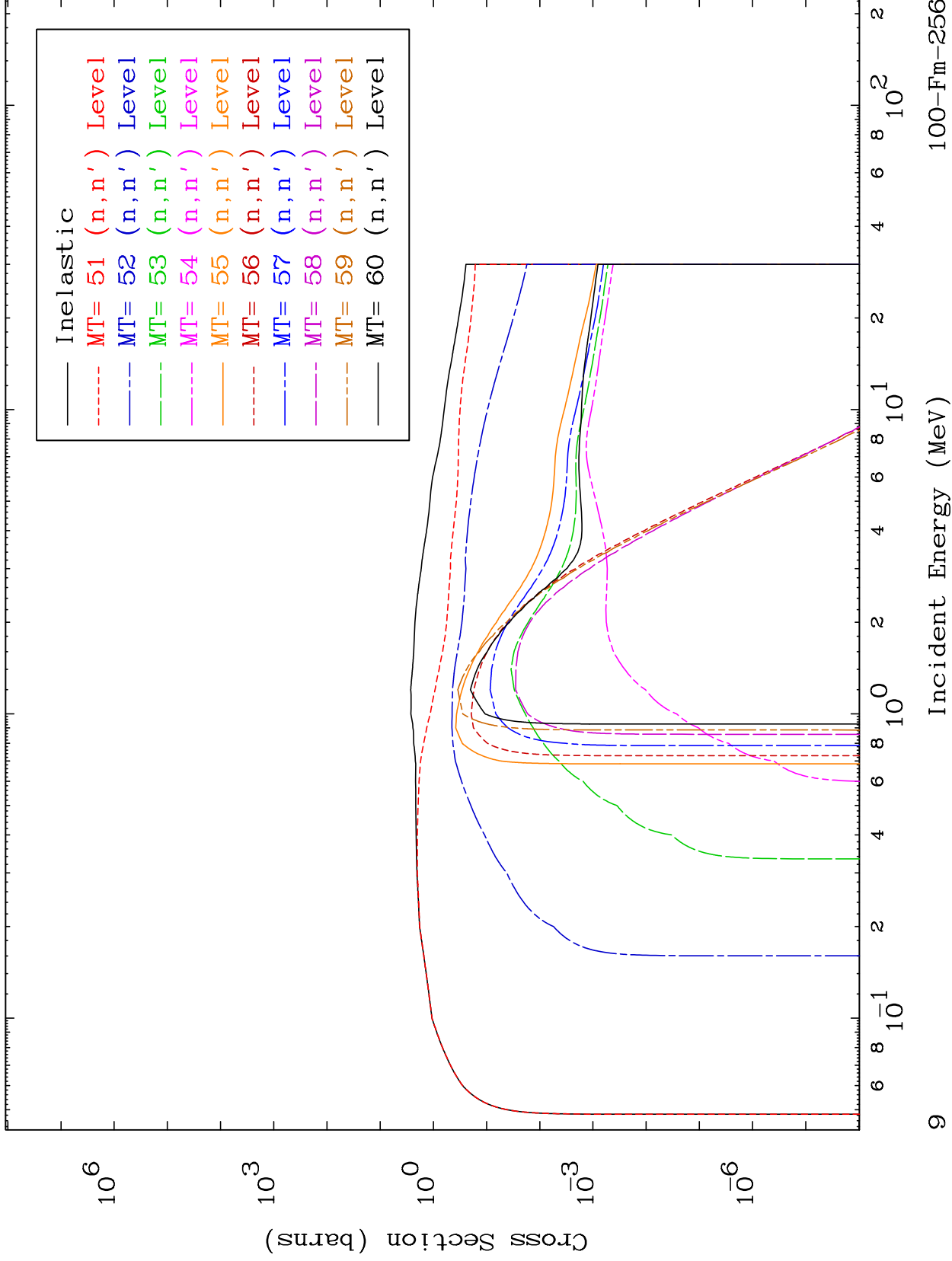


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(n,n') Levels

100-Fm-256

293 Kelvin Cross Sections

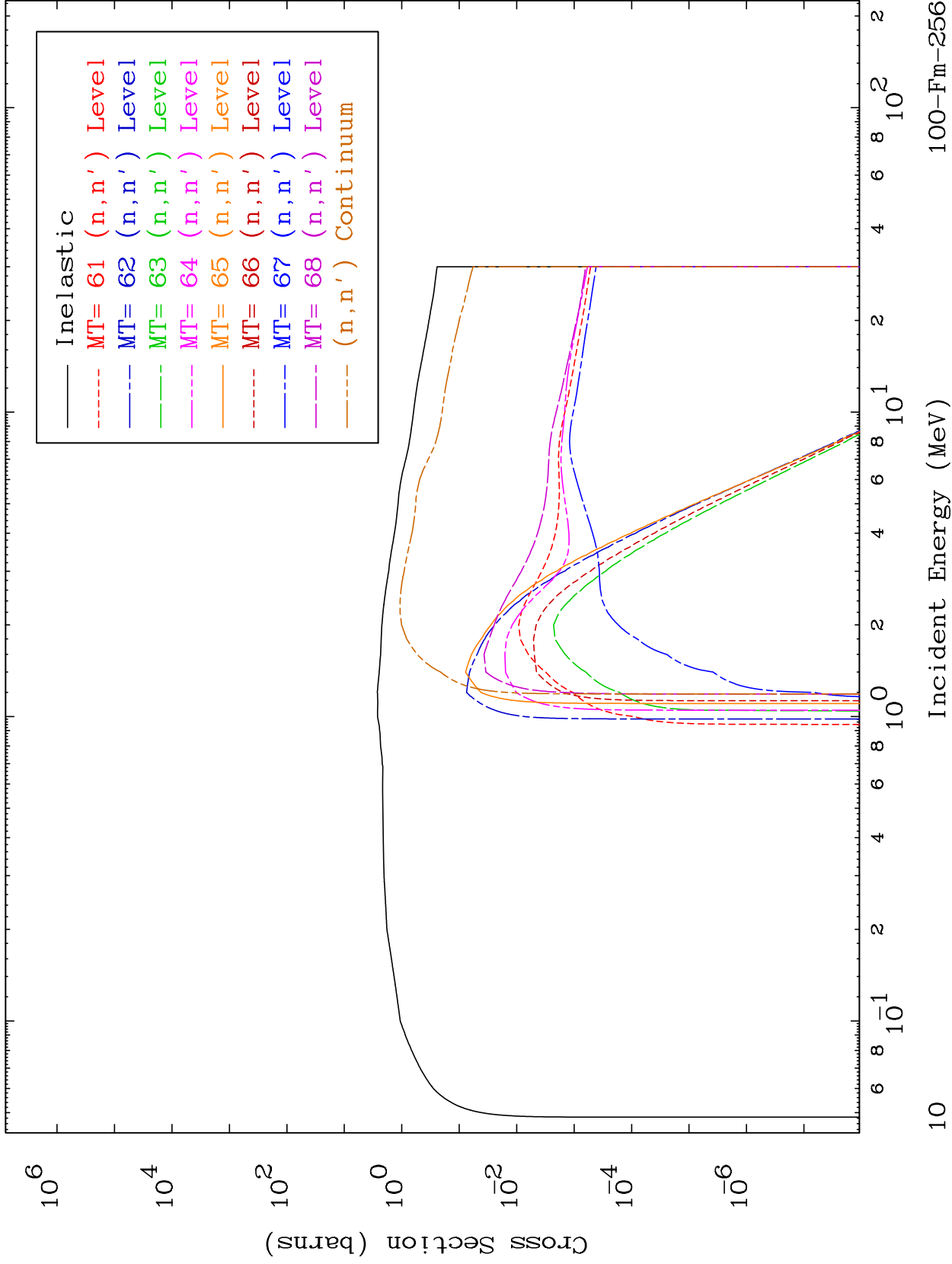


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(n,n') Levels

293 Kelvin Cross Sections

100-Fm-256



10

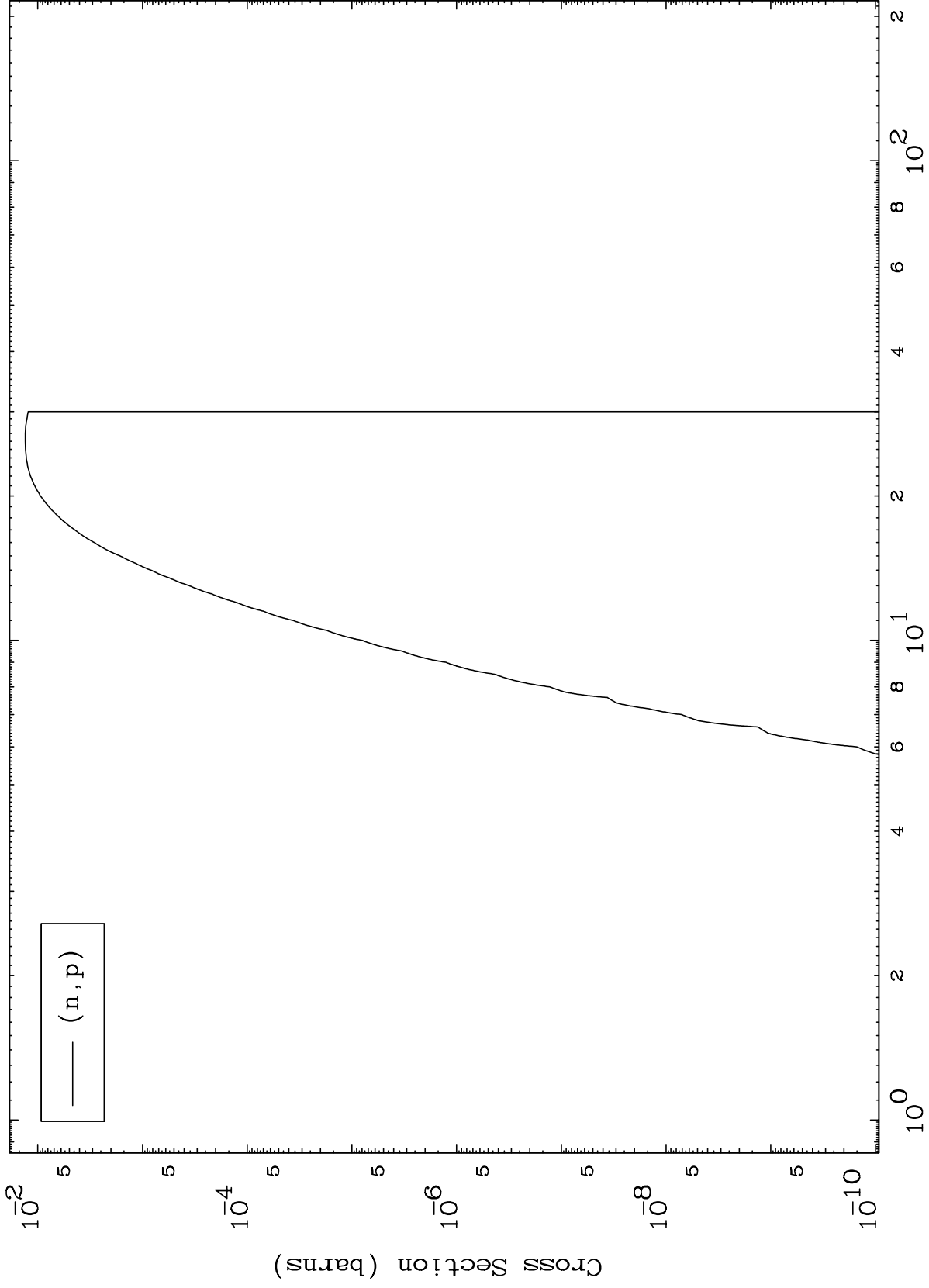
Incident Energy (MeV)

100-Fm-256

MAT 9937

(n,p) Levels
293 Kelvin Cross Sections

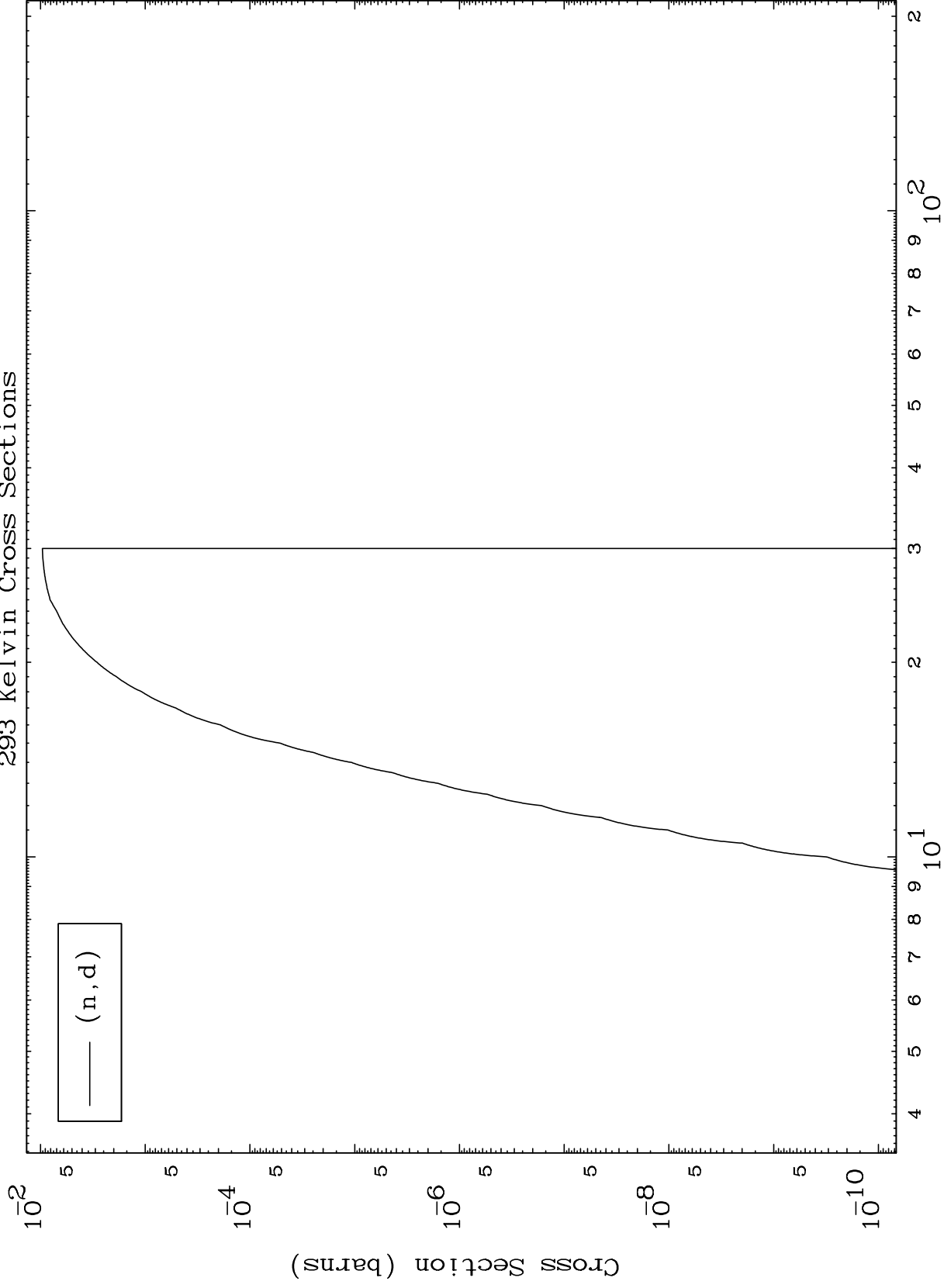
100-Fm-256



MAT 9937

(n,d) Levels
293 Kelvin Cross Sections

100-Fm-256



12

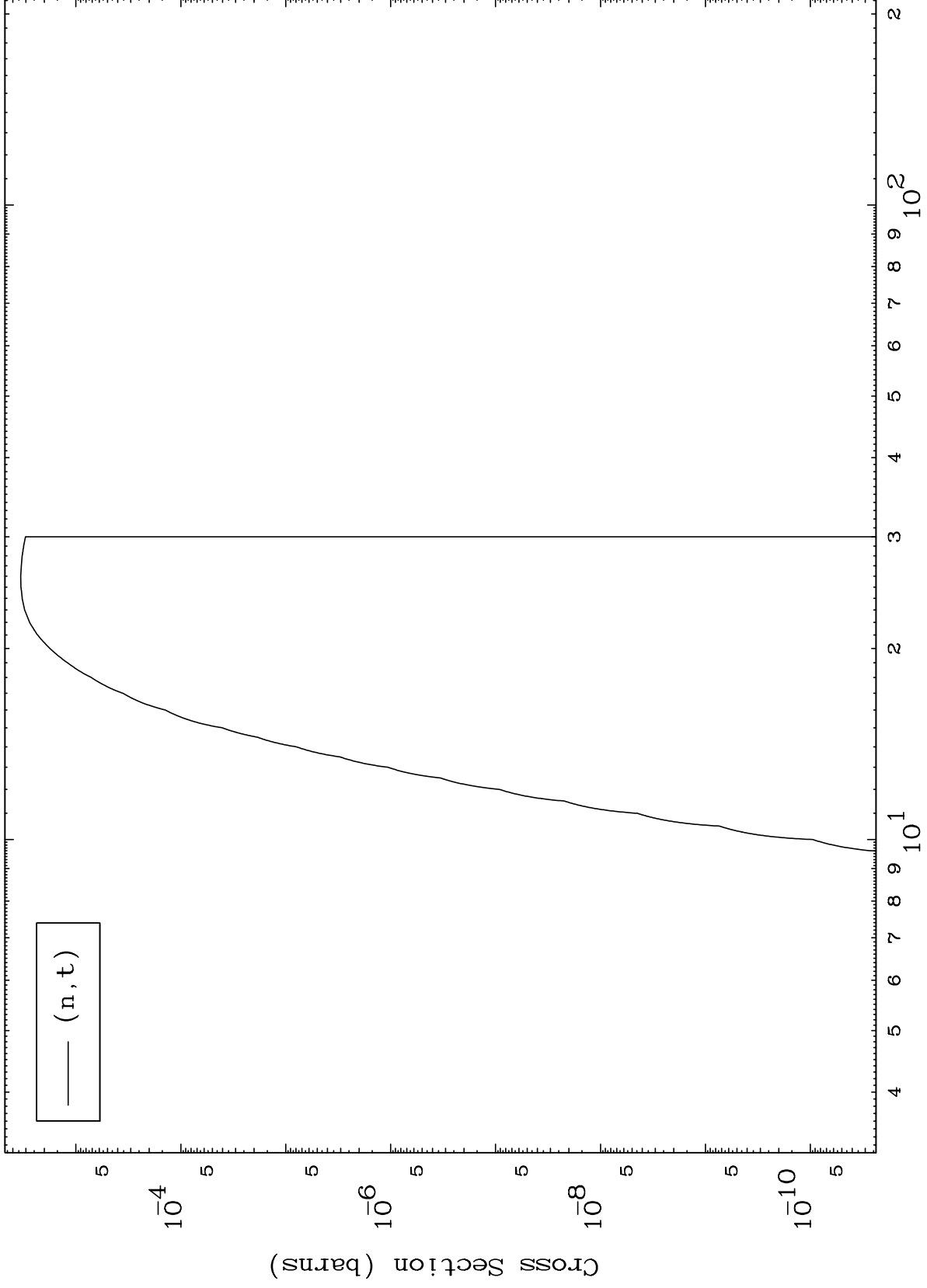
Incident Energy (MeV)

100-Fm-256

MAT 9937

(n,t) Levels
293 Kelvin Cross Sections

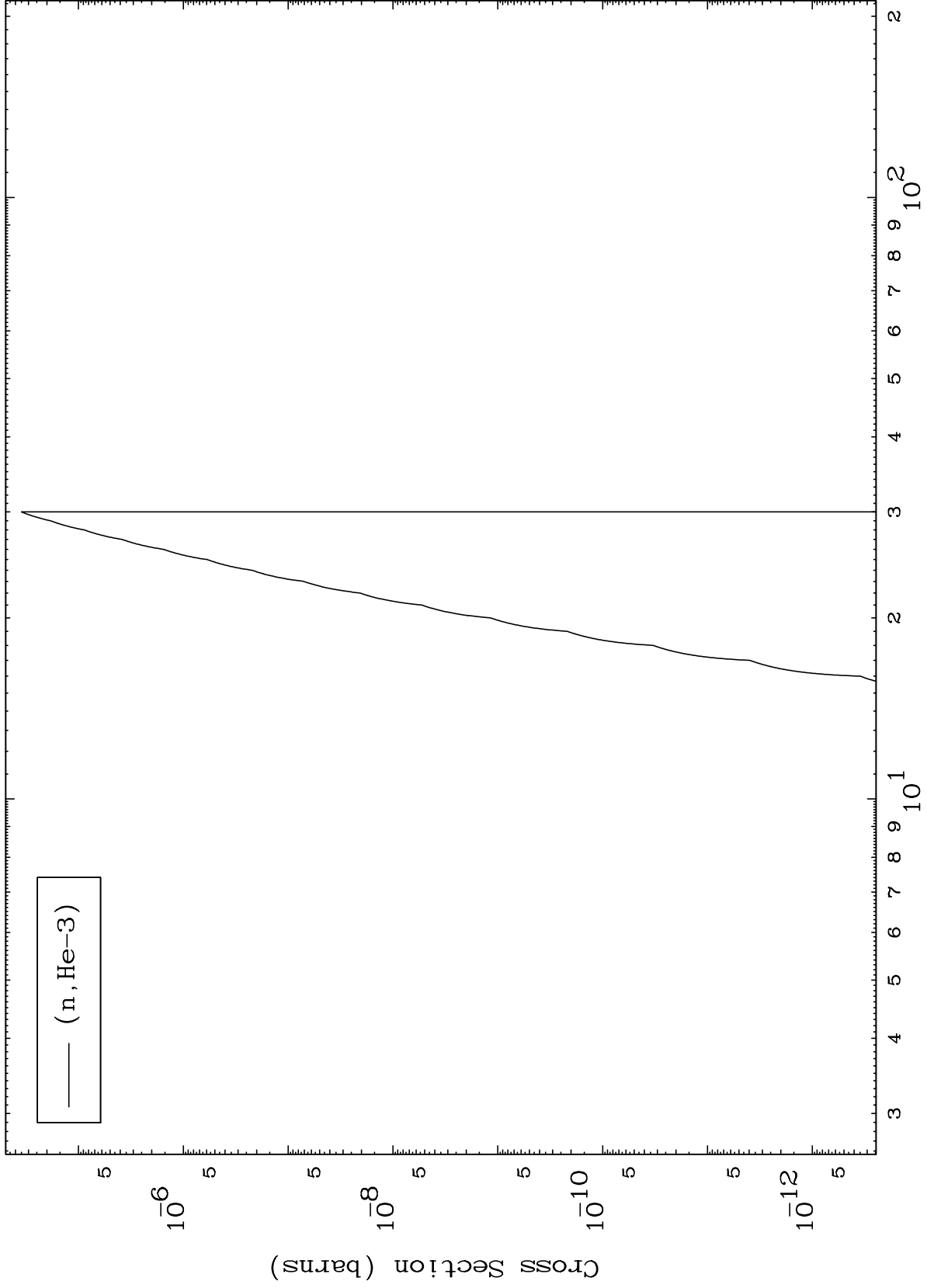
100-Fm-256



MAT 9937

(n,He3) Levels
293 Kelvin Cross Sections

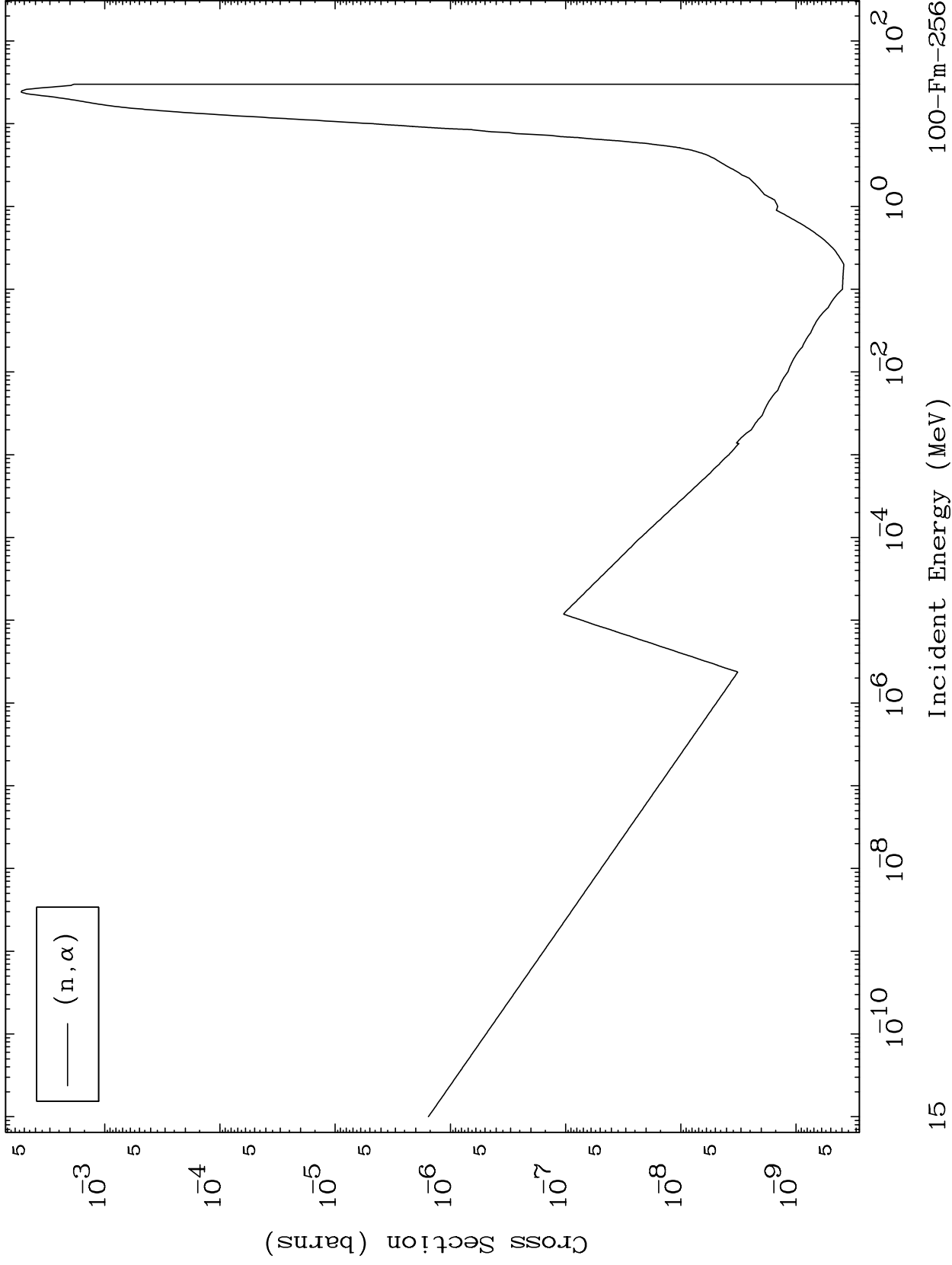
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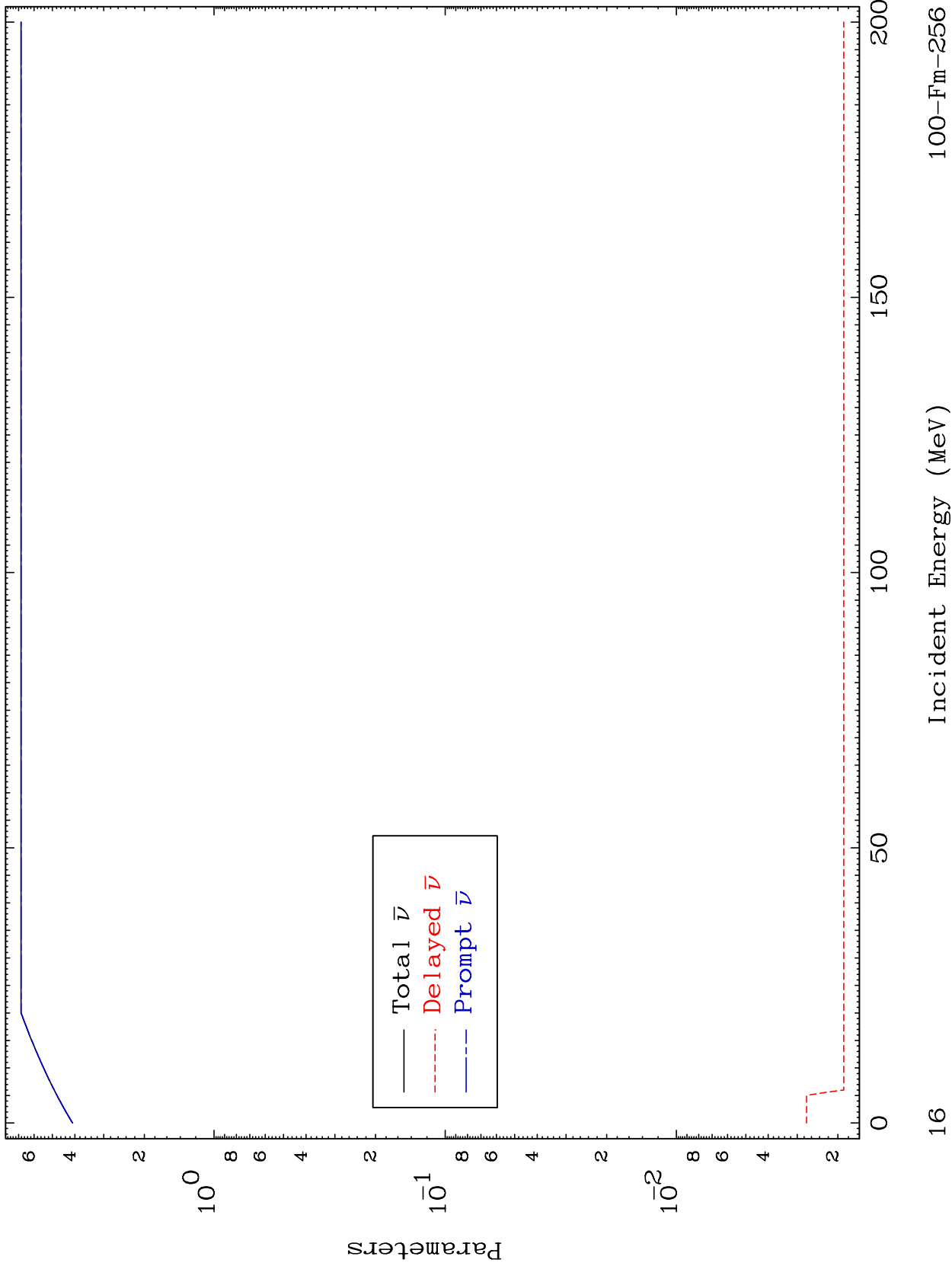


MAT 9937

(n, α) Levels
293 Kelvin Cross Sections

100-Fm-256

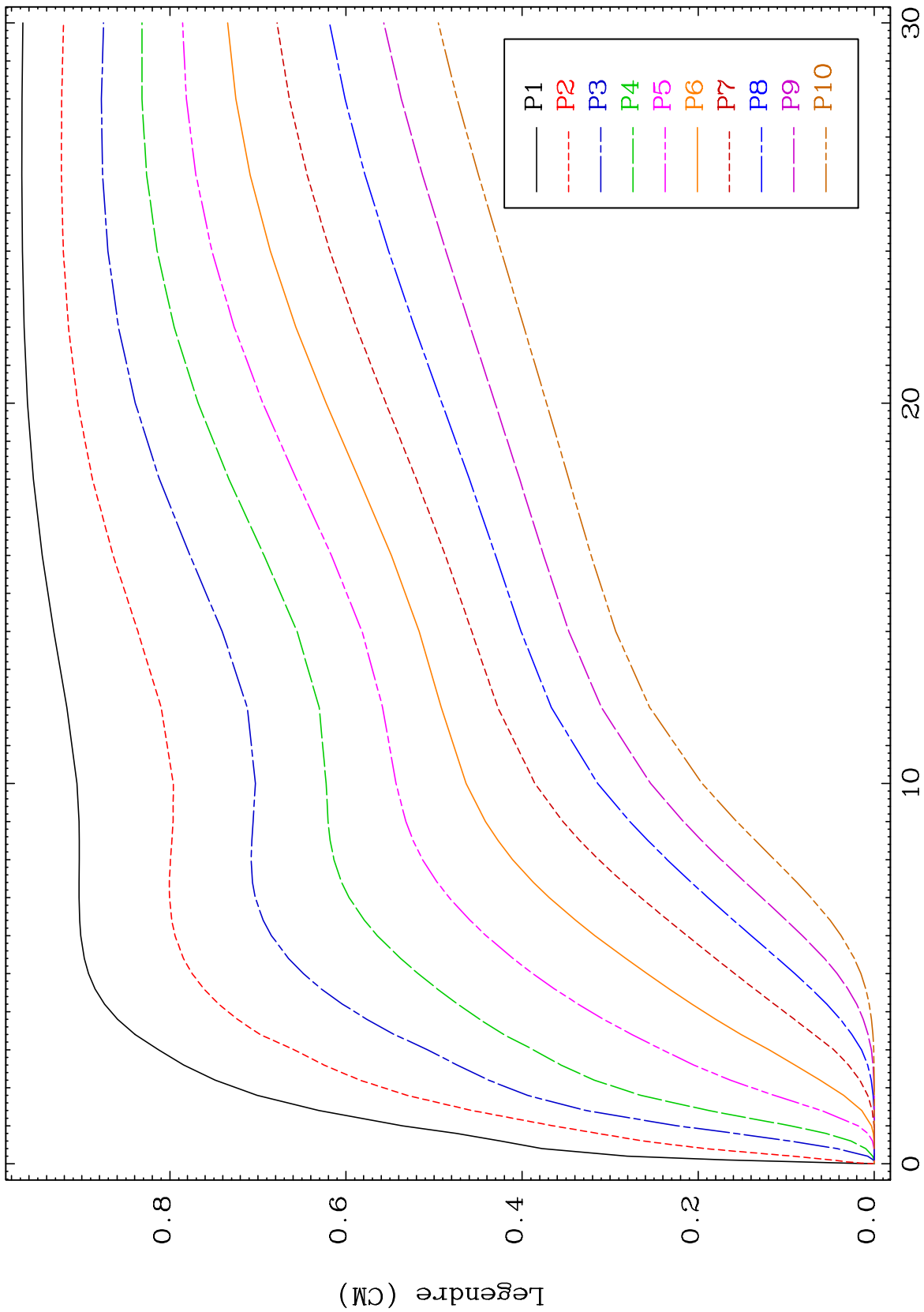




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Elastic Legendre Coefficients

100-Fm-256



17

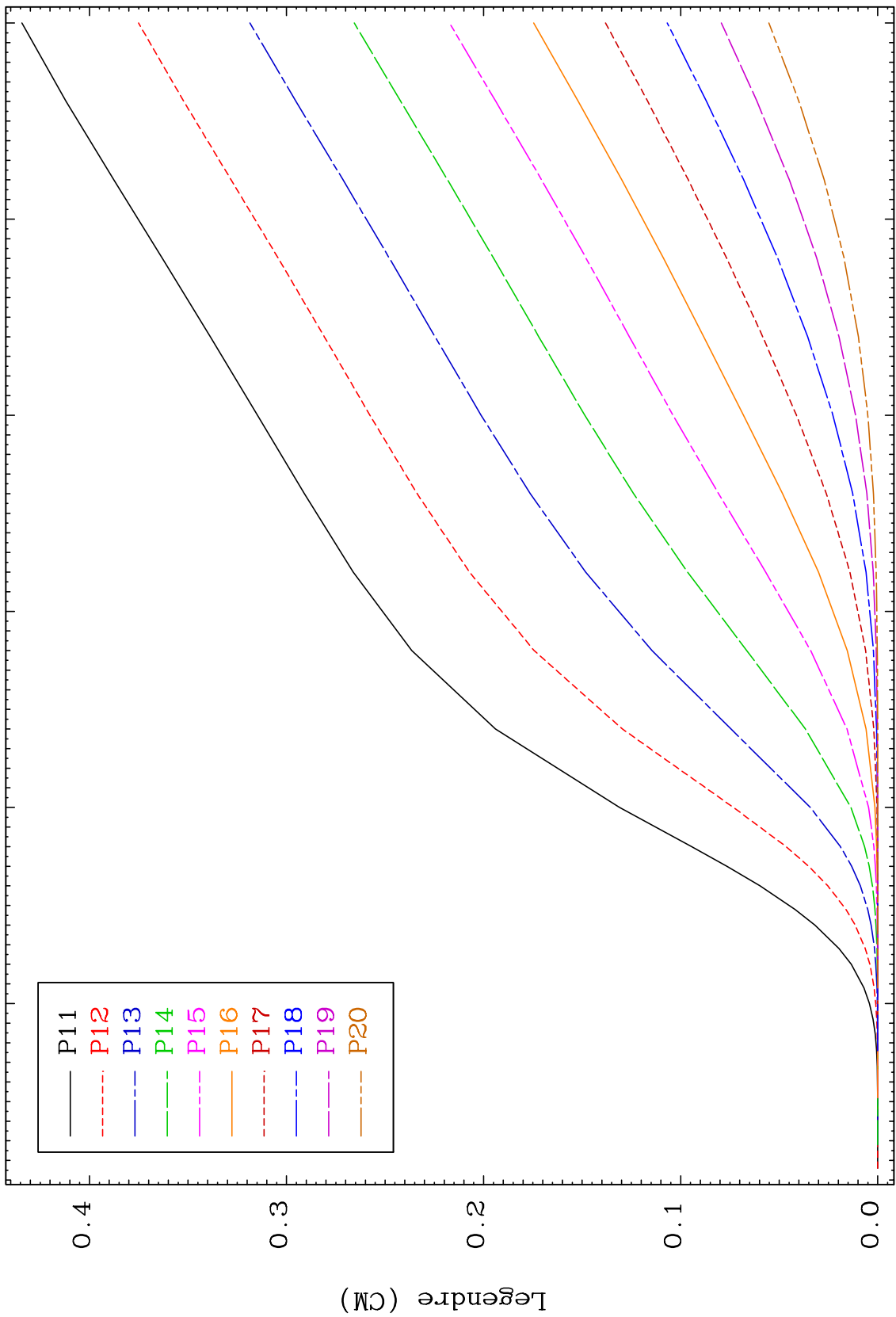
Incident Energy (MeV)

100-Fm-256

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Elastic Legendre Coefficients

100-Fm-256



18

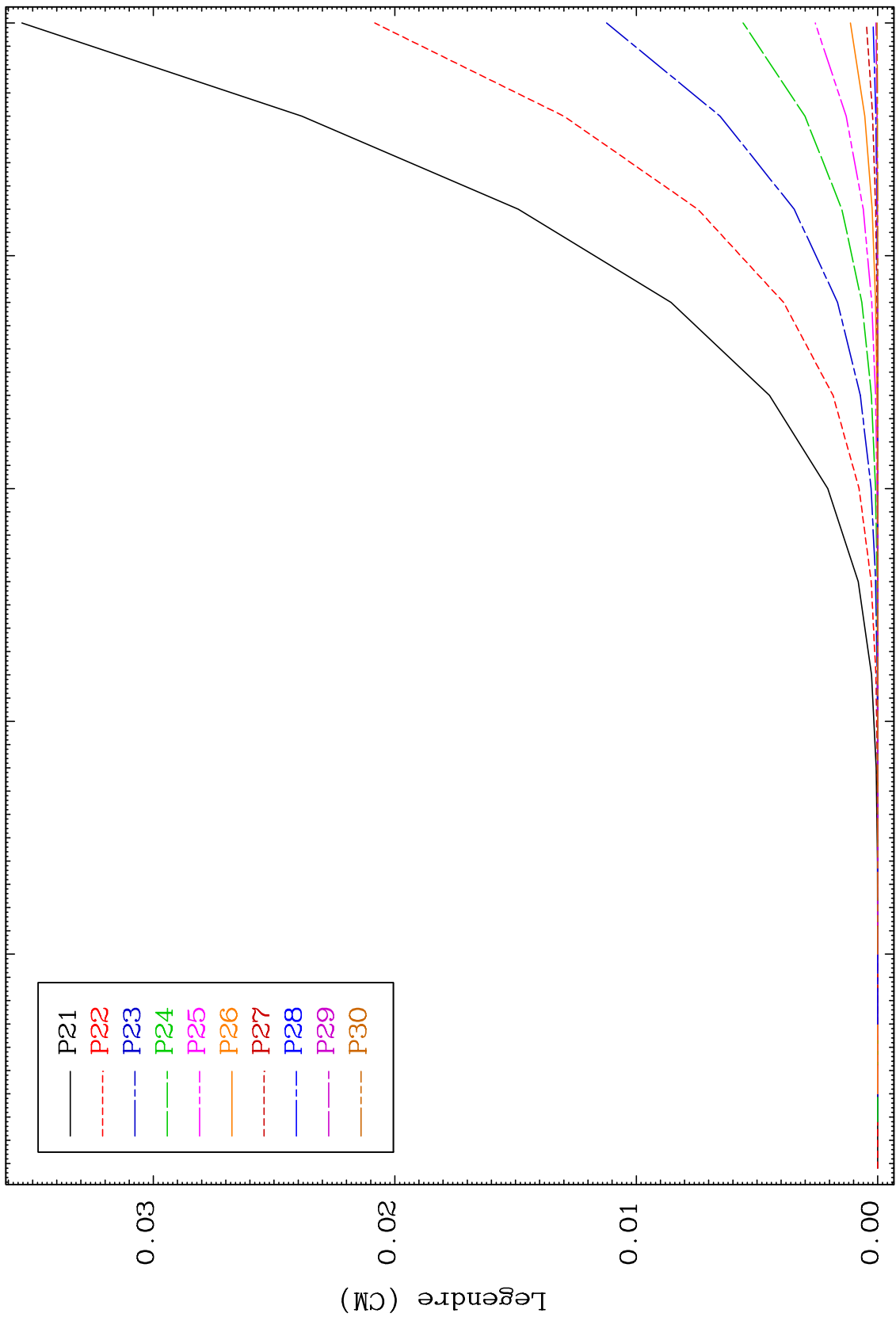
Incident Energy (MeV)

100-Fm-256

MAT 9937

Elastic
Legendre Coefficients

100-Fm-256



19

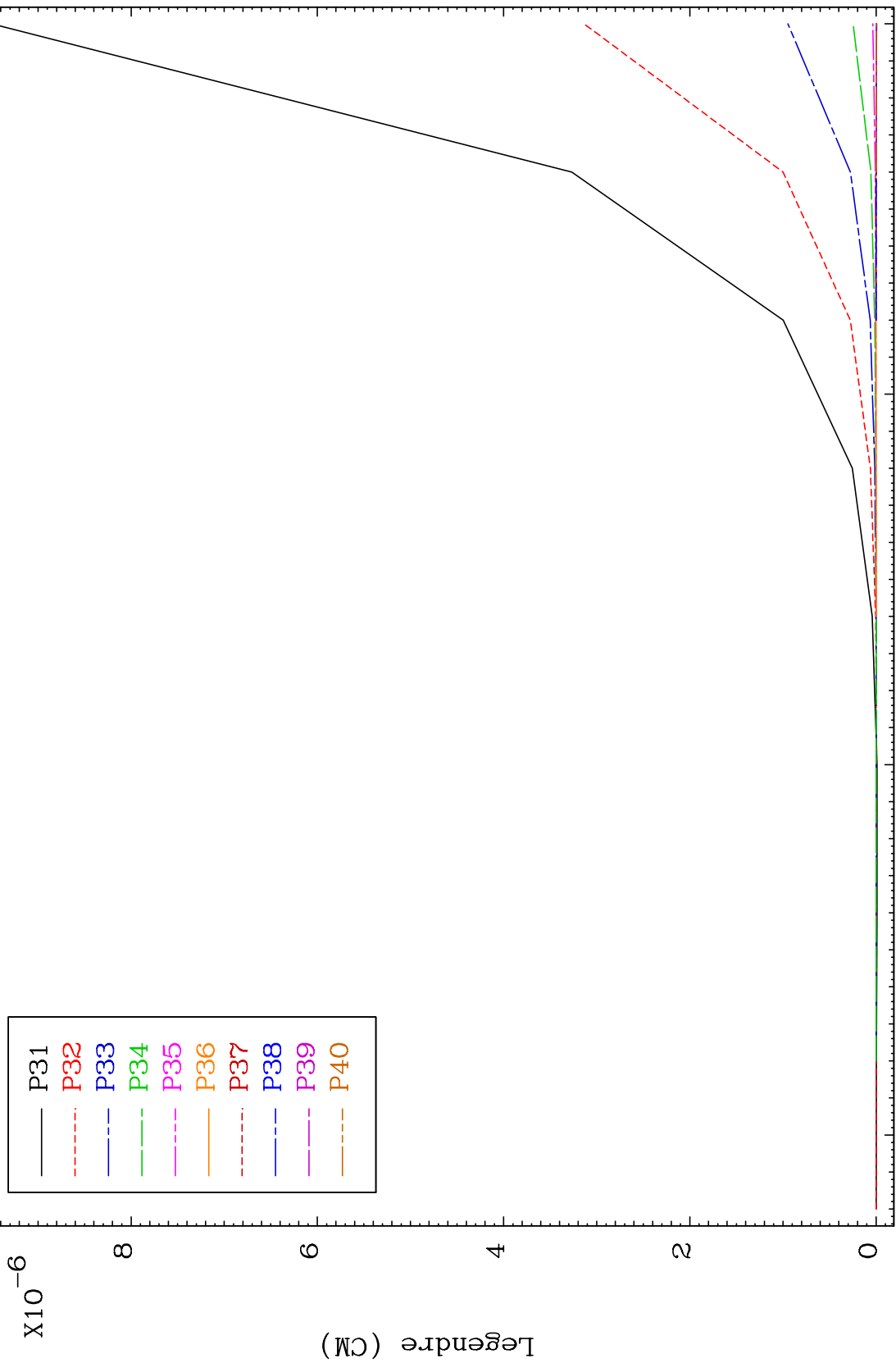
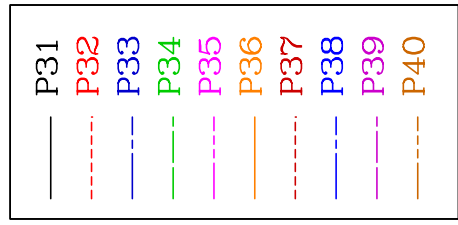
Incident Energy (MeV)

100-Fm-256

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Elastic Legendre Coefficients

100-Fm-256



20

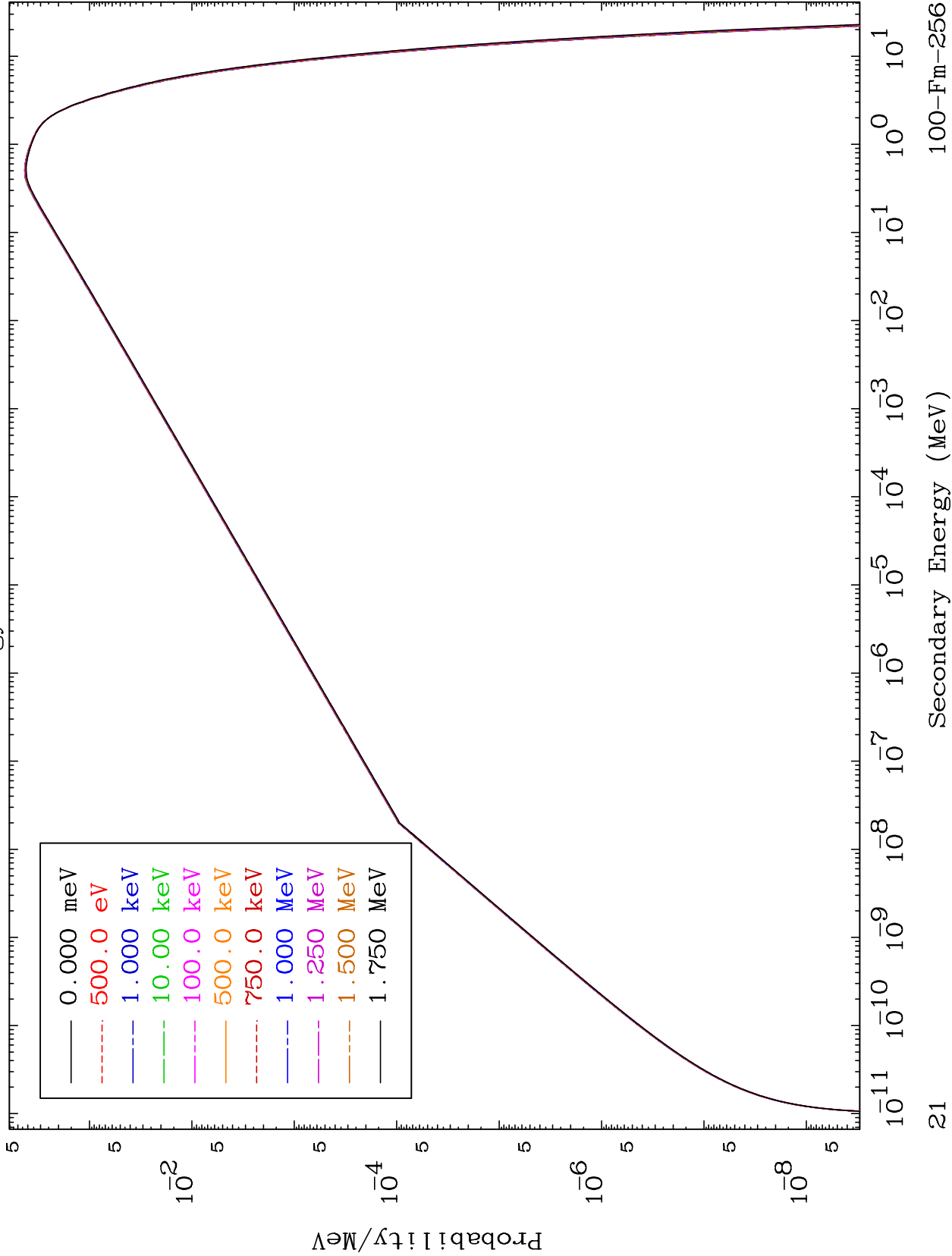
Incident Energy (MeV)

100-Fm-256

MAT 9937

Fission Energy Distributions

100-Fm-256



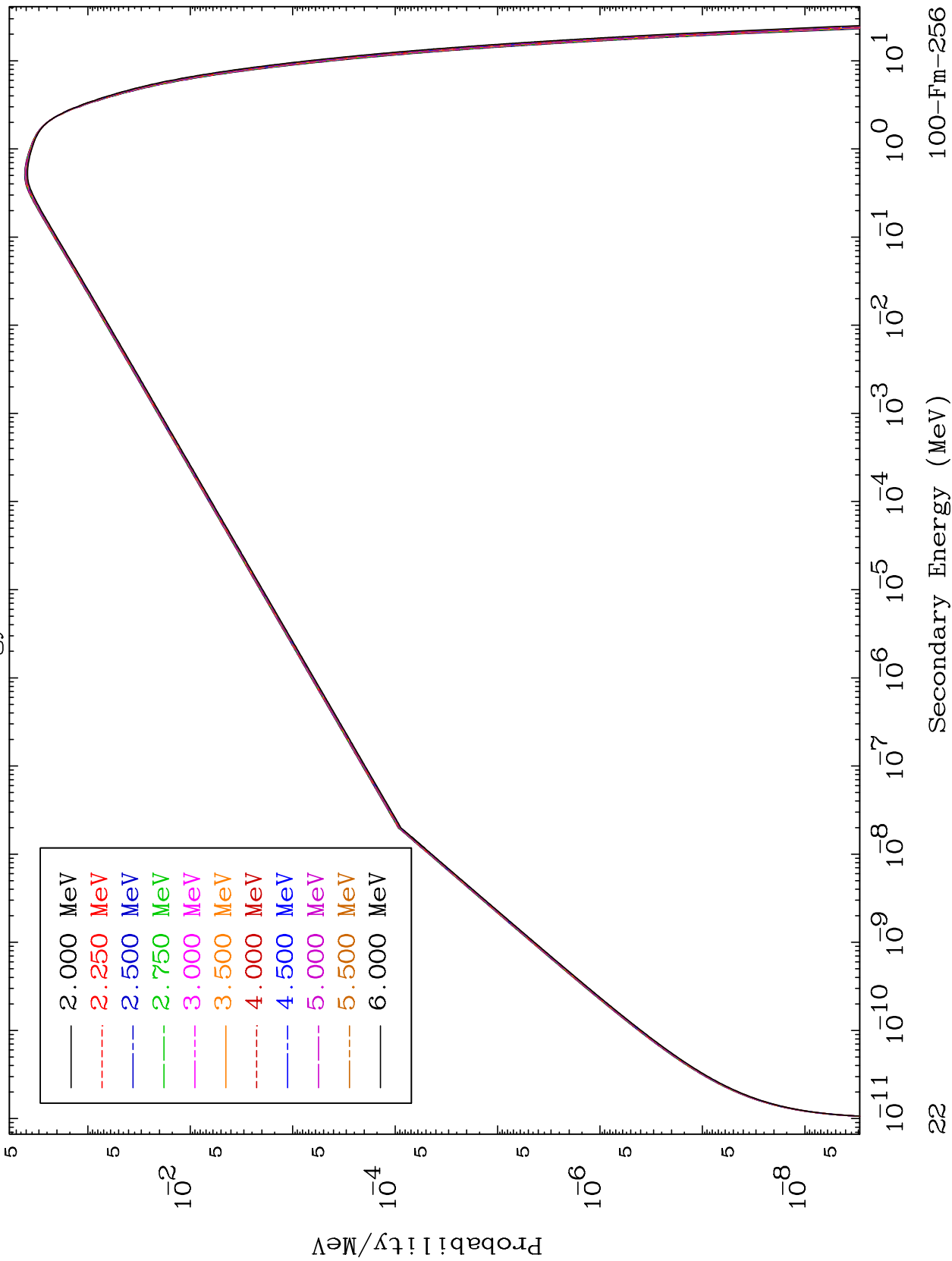
100-Fm-256

Secondary Energy (MeV)

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Fission Energy Distributions

100-Fm-256



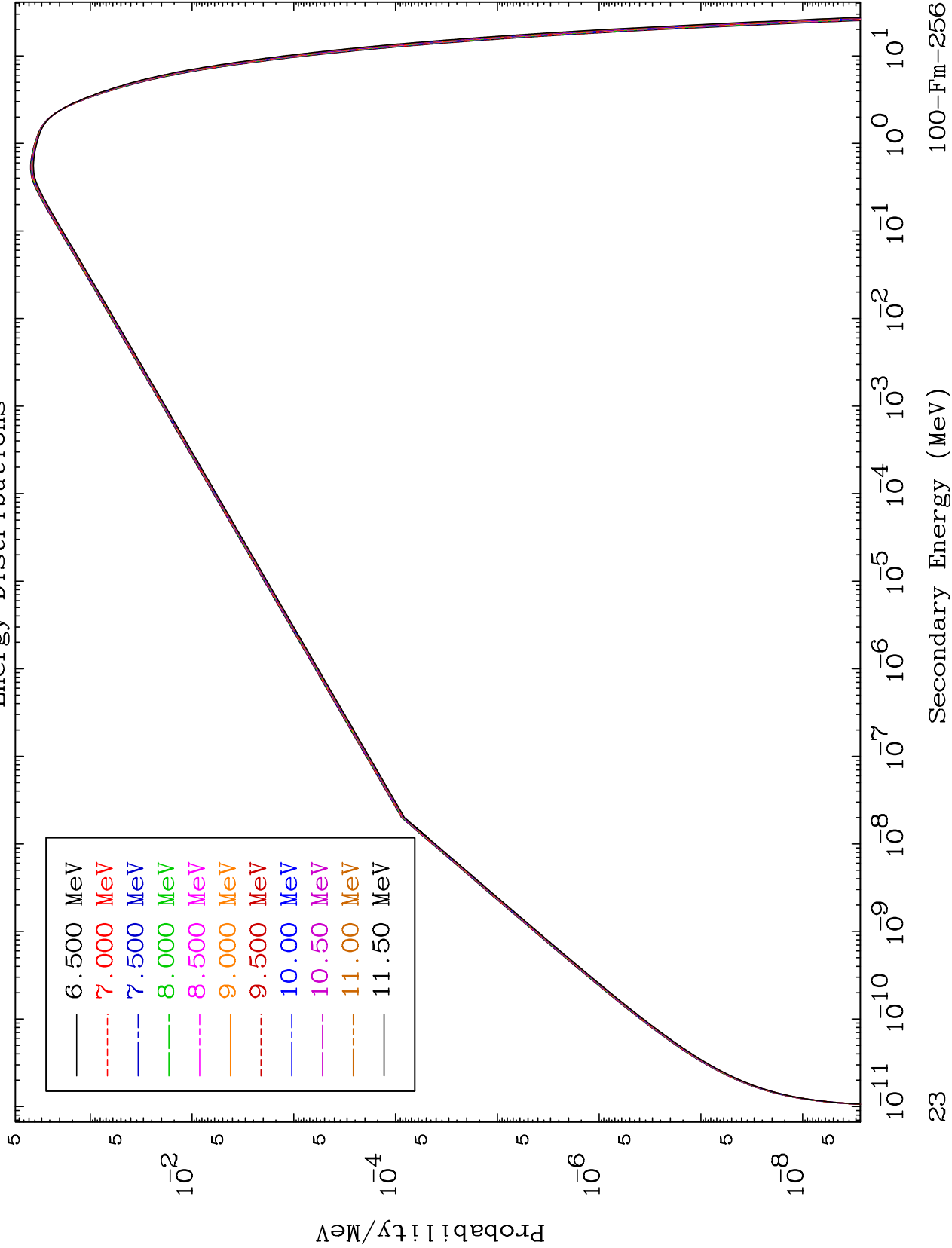
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Secondary Energy (MeV)

MAT 9937

Fission Energy Distributions

100-Fm-256



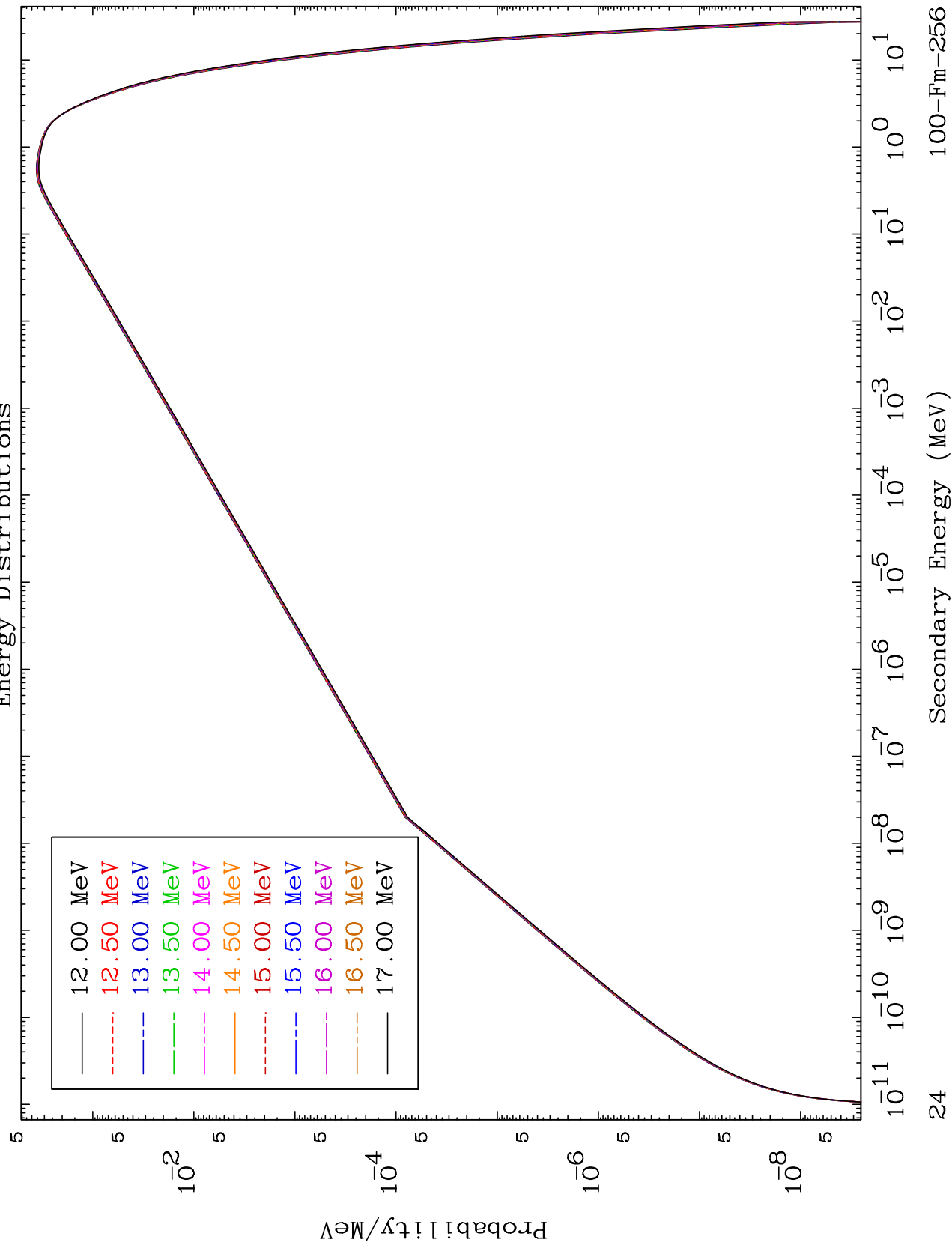
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Secondary Energy (MeV)

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Fission Energy Distributions

100-Fm-256



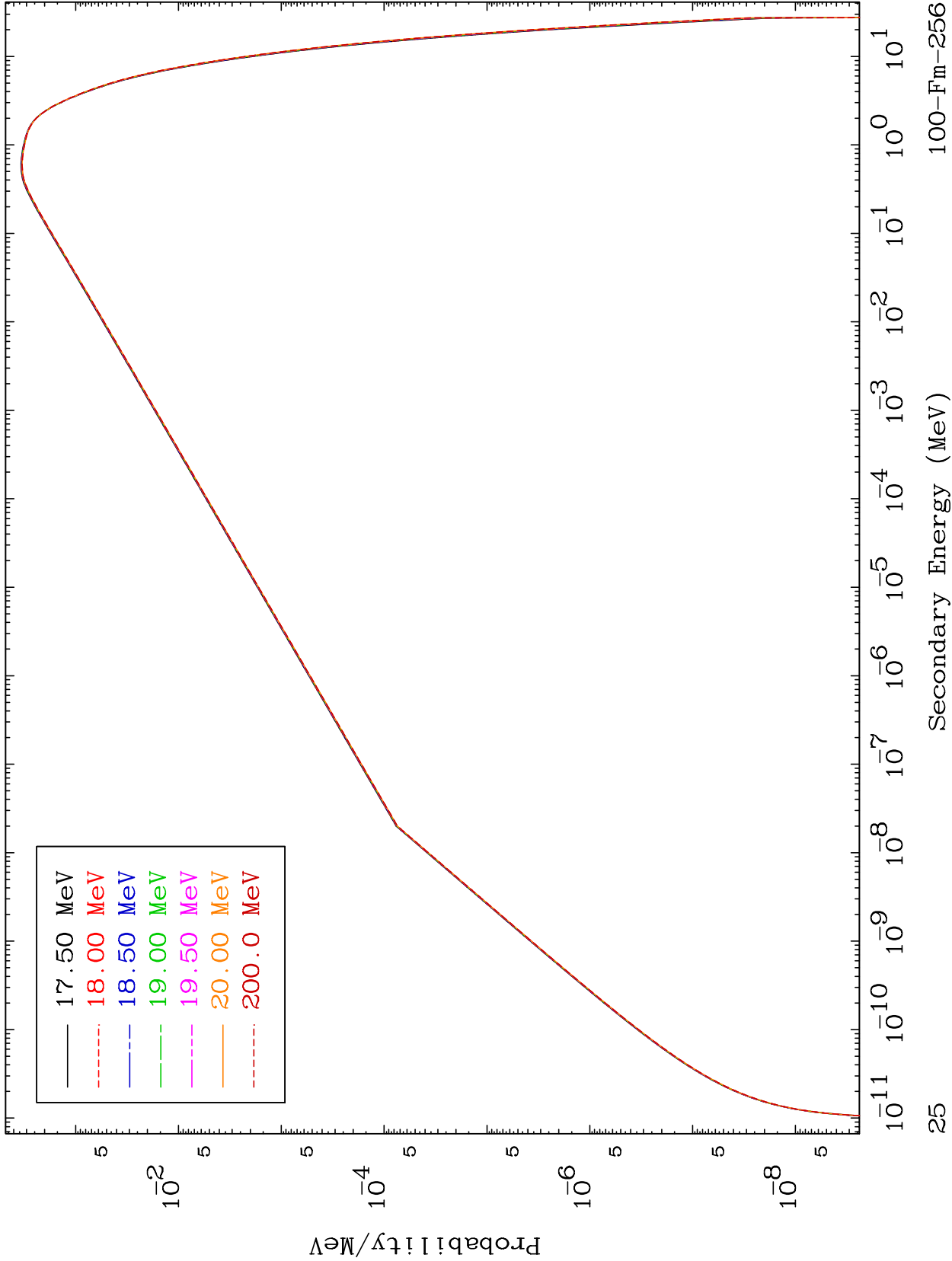
100-Fm-256

Secondary Energy (MeV)

MAT 9937

Fission Energy Distributions

100-Fm-256

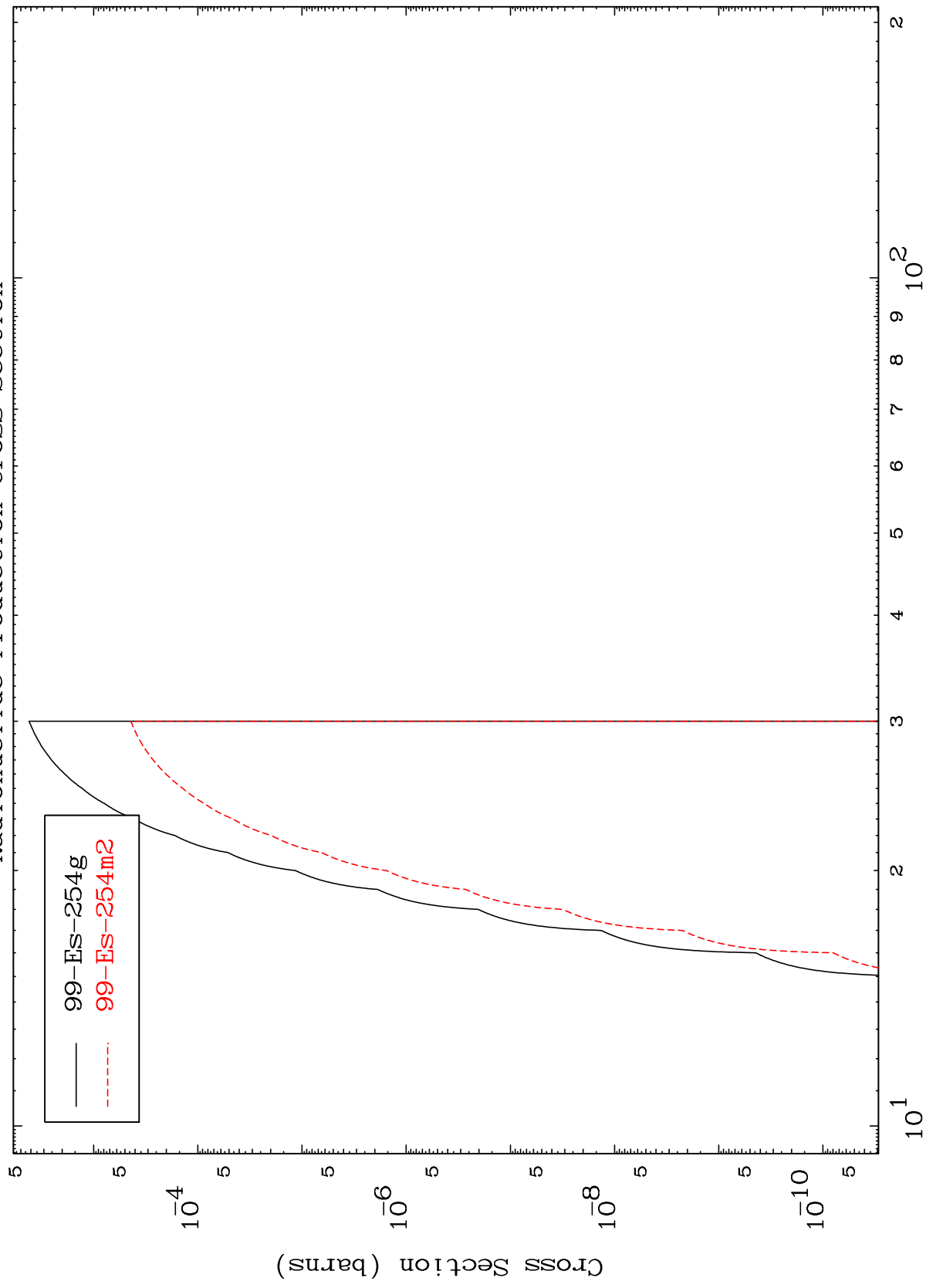


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(n,n') d

100-Fm-256

Radionuclide Production Cross Section



26

Incident Energy (MeV)

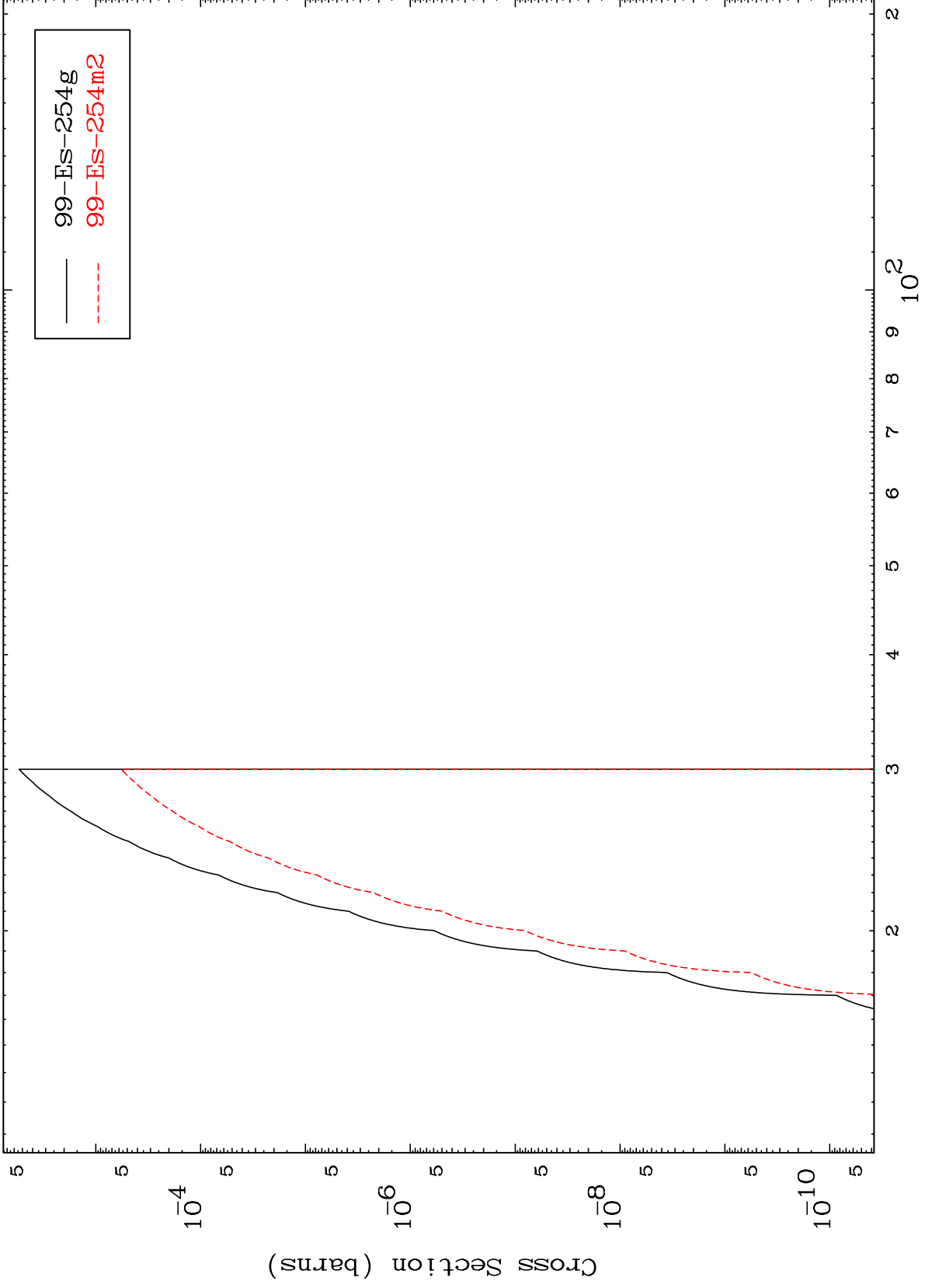
100-Fm-256

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(n,2n) p

100-Fm-256

Radionuclide Production Cross Section



27

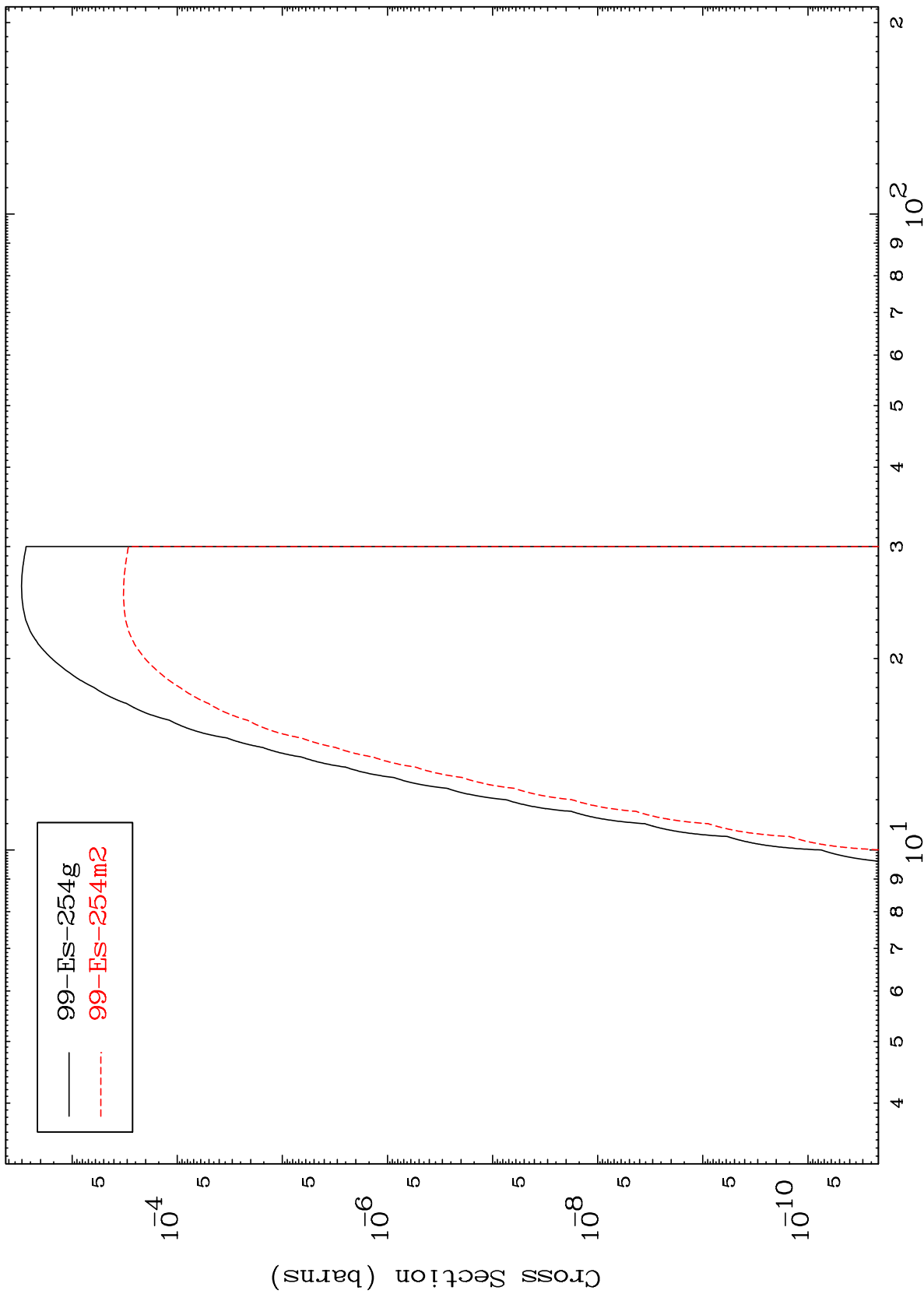
Incident Energy (MeV)

100-Fm-256

MAT 9937

100-Fm-256

(n, t)
Radionuclide Production Cross Section



100-Fm-256

Incident Energy (MeV)

28