

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

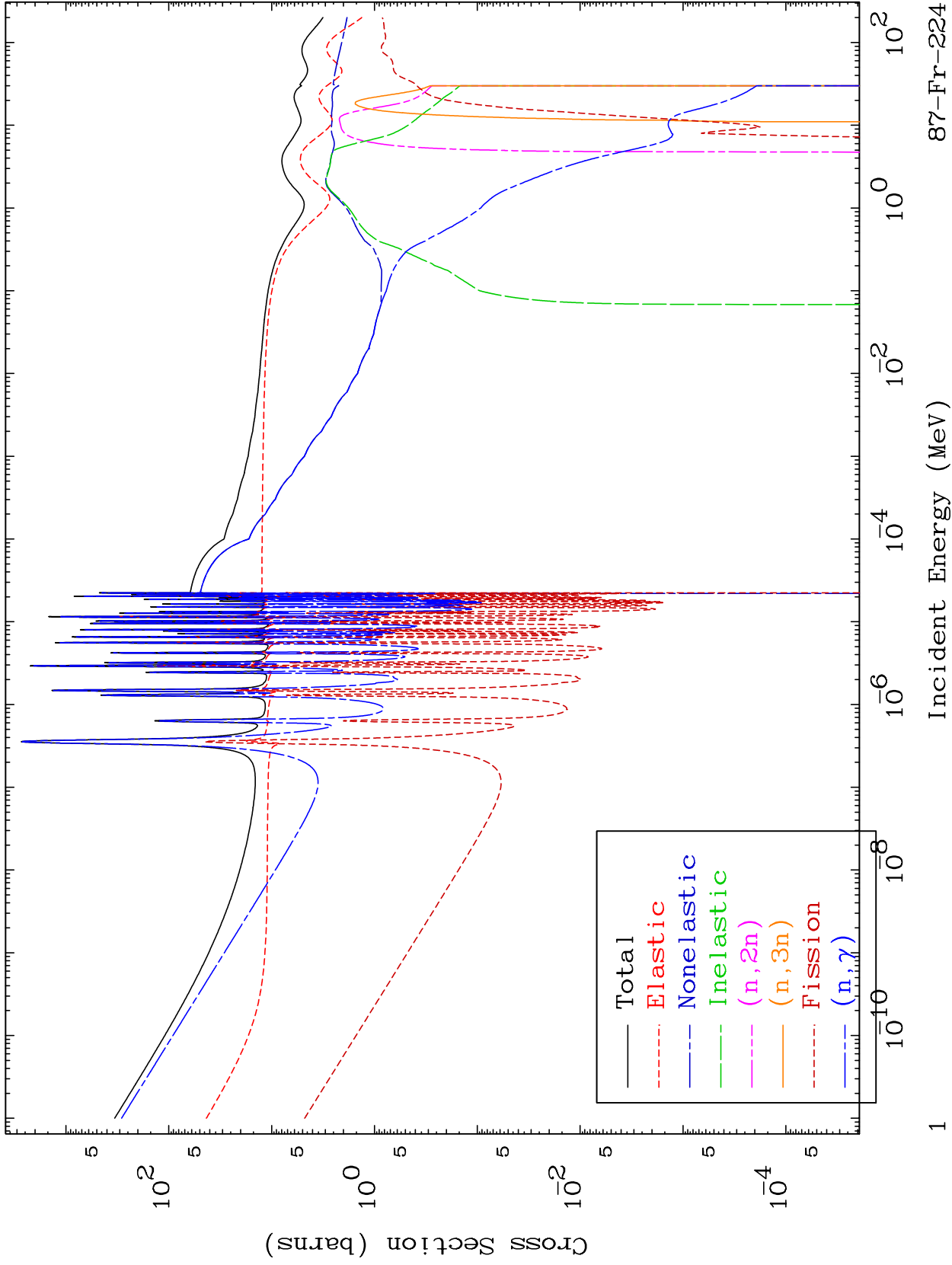
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8761

Neutron Major
293 Kelvin Cross Sections

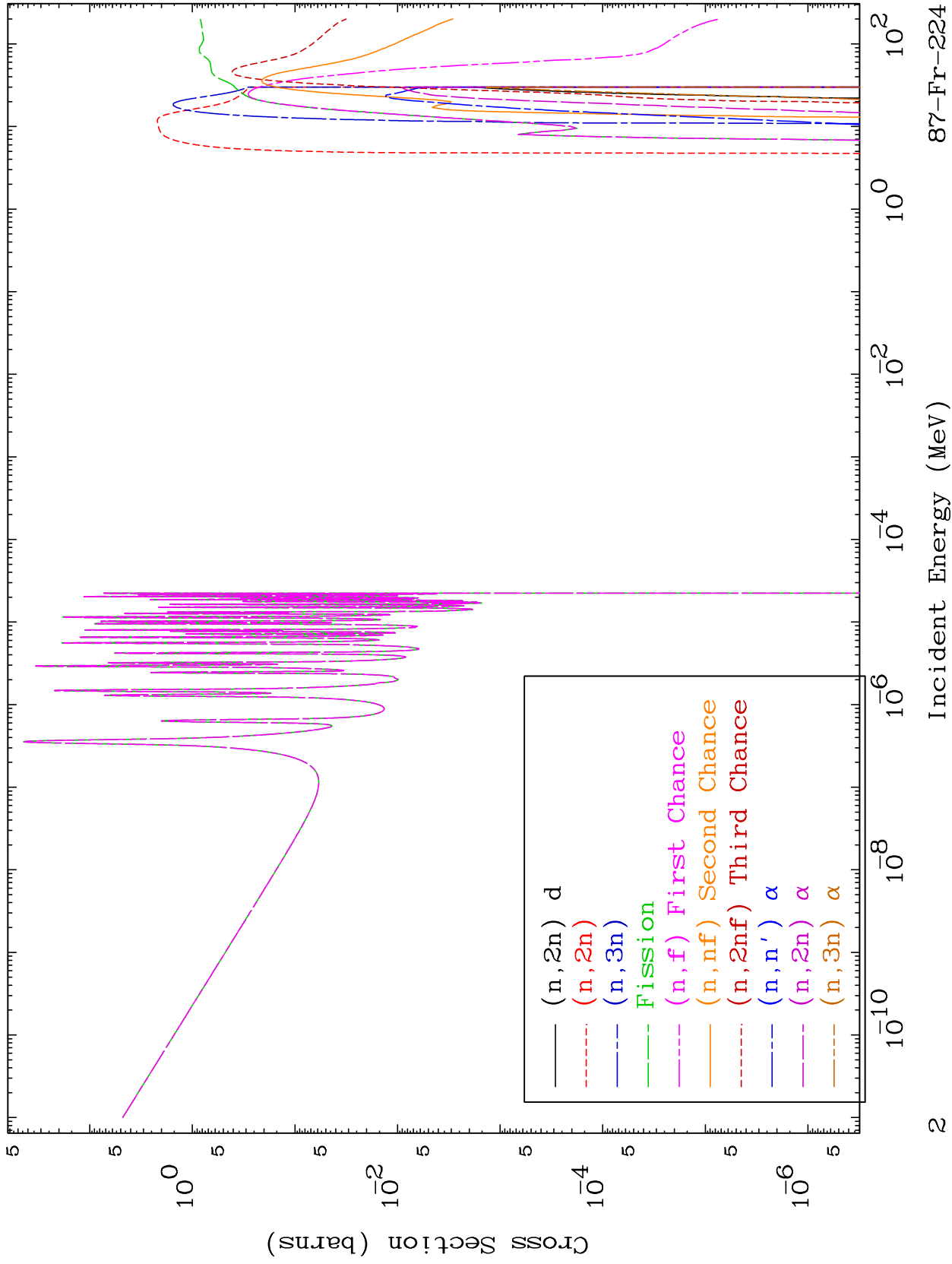
87-Fr-224



MAT 8761

Neutron Absorption
293 Kelvin Cross Sections

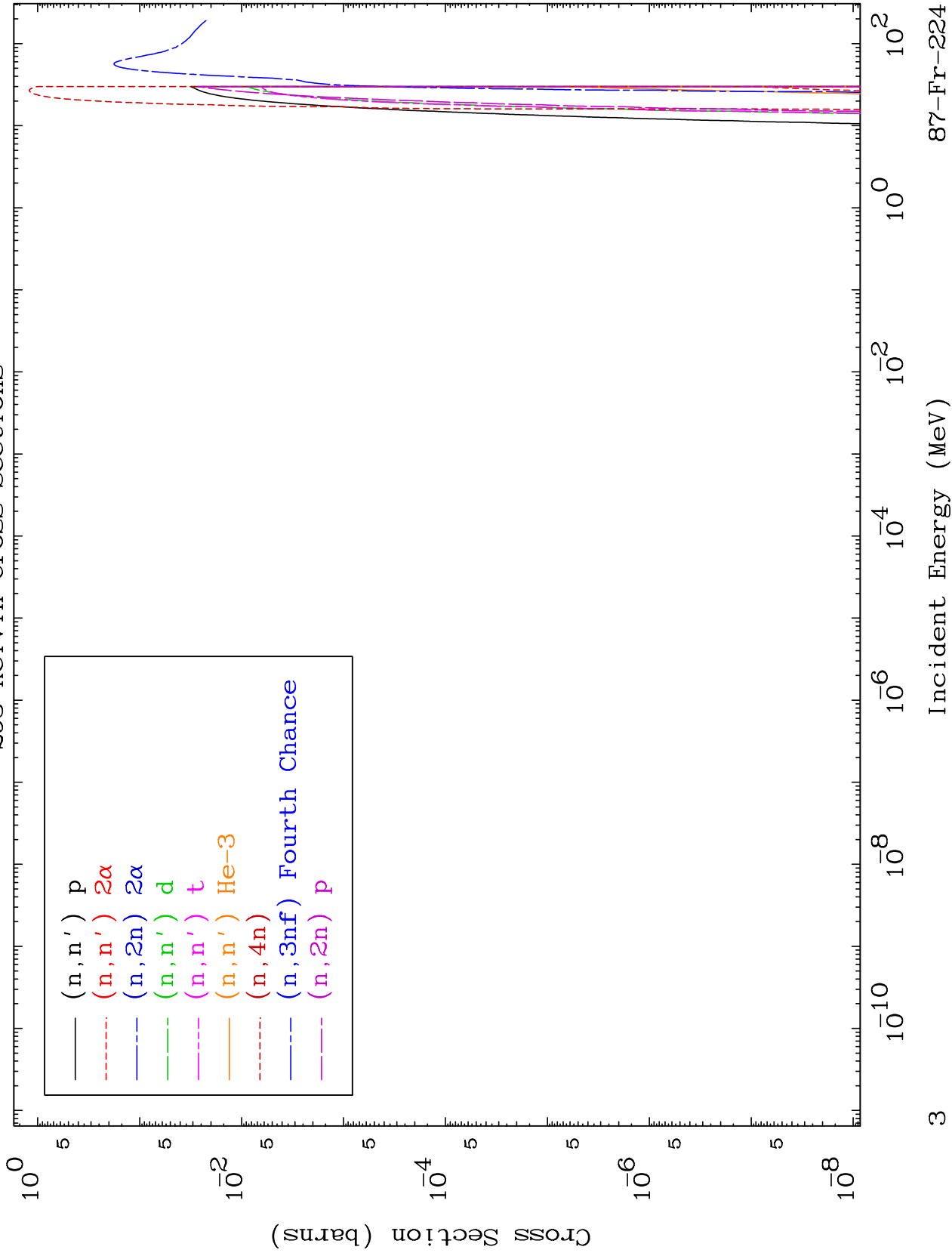
87-Fr-224



MAT 8761

Neutron Absorption
293 Kelvin Cross Sections

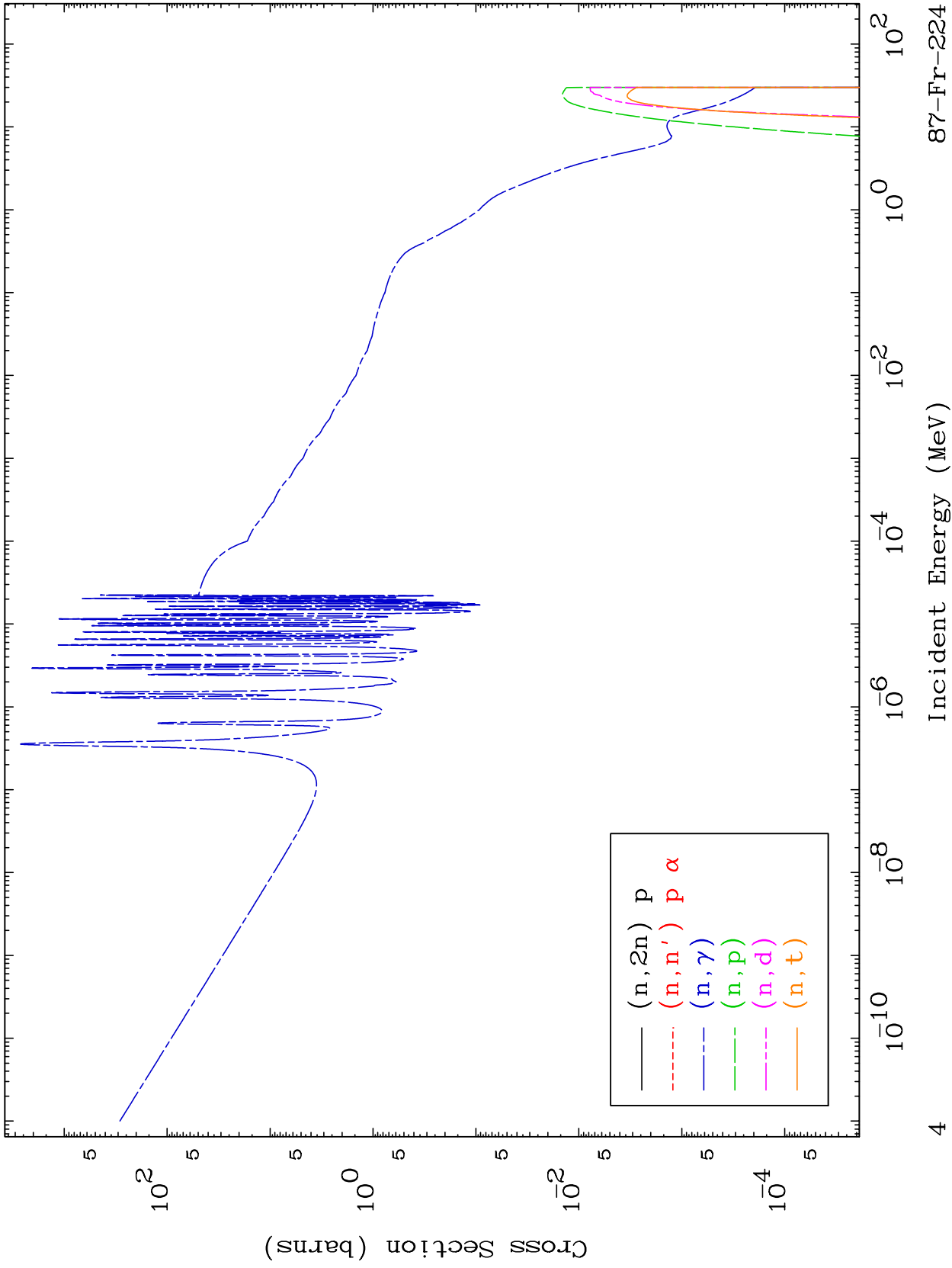
87-Fr-224



MAT 8761

Neutron Absorption
293 Kelvin Cross Sections

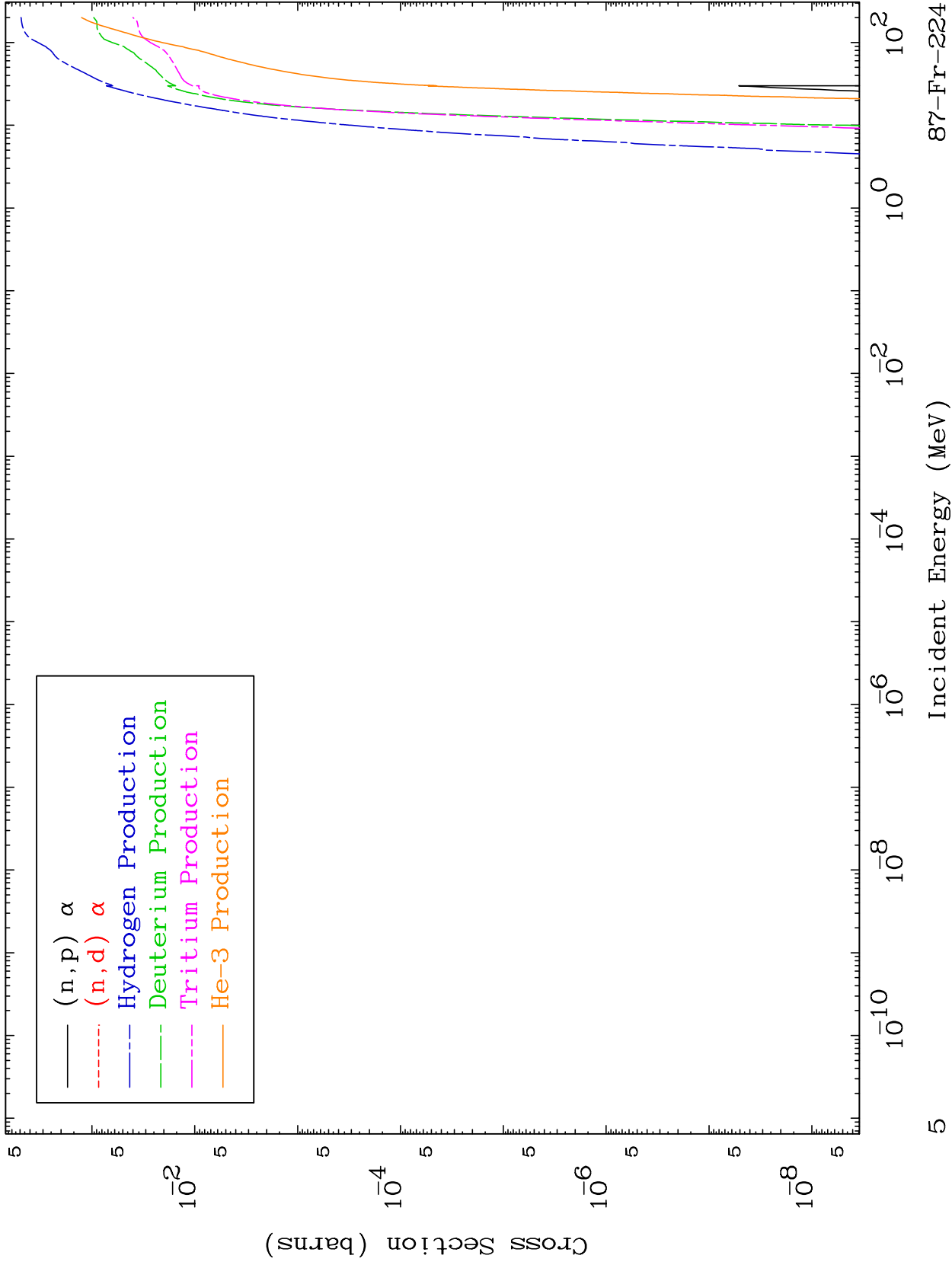
87-Fr-224



MAT 8761

Neutron Absorption
293 Kelvin Cross Sections

87-Fr-224

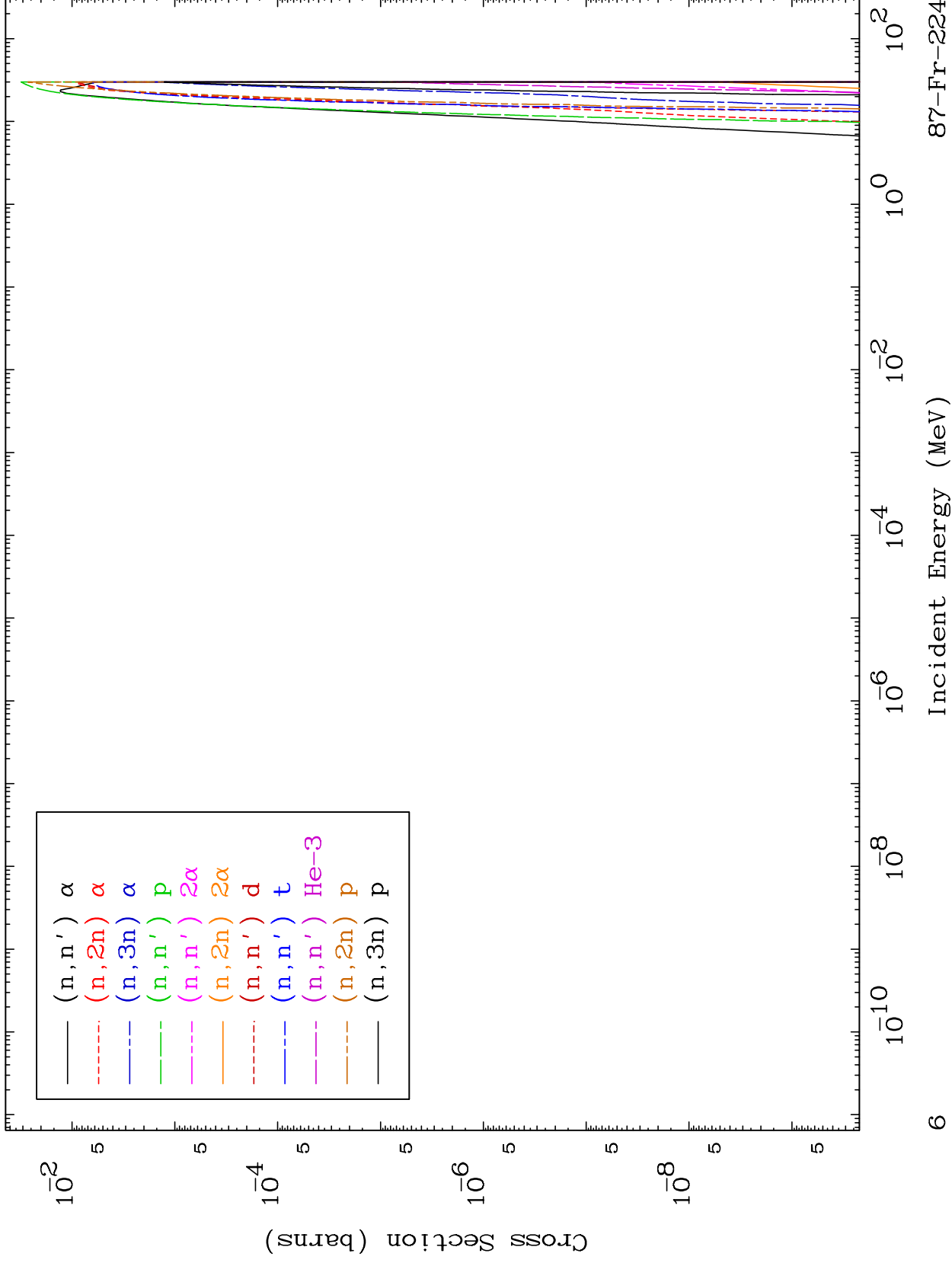


5

MAT 8761

Charged Particle
293 Kelvin Cross Sections

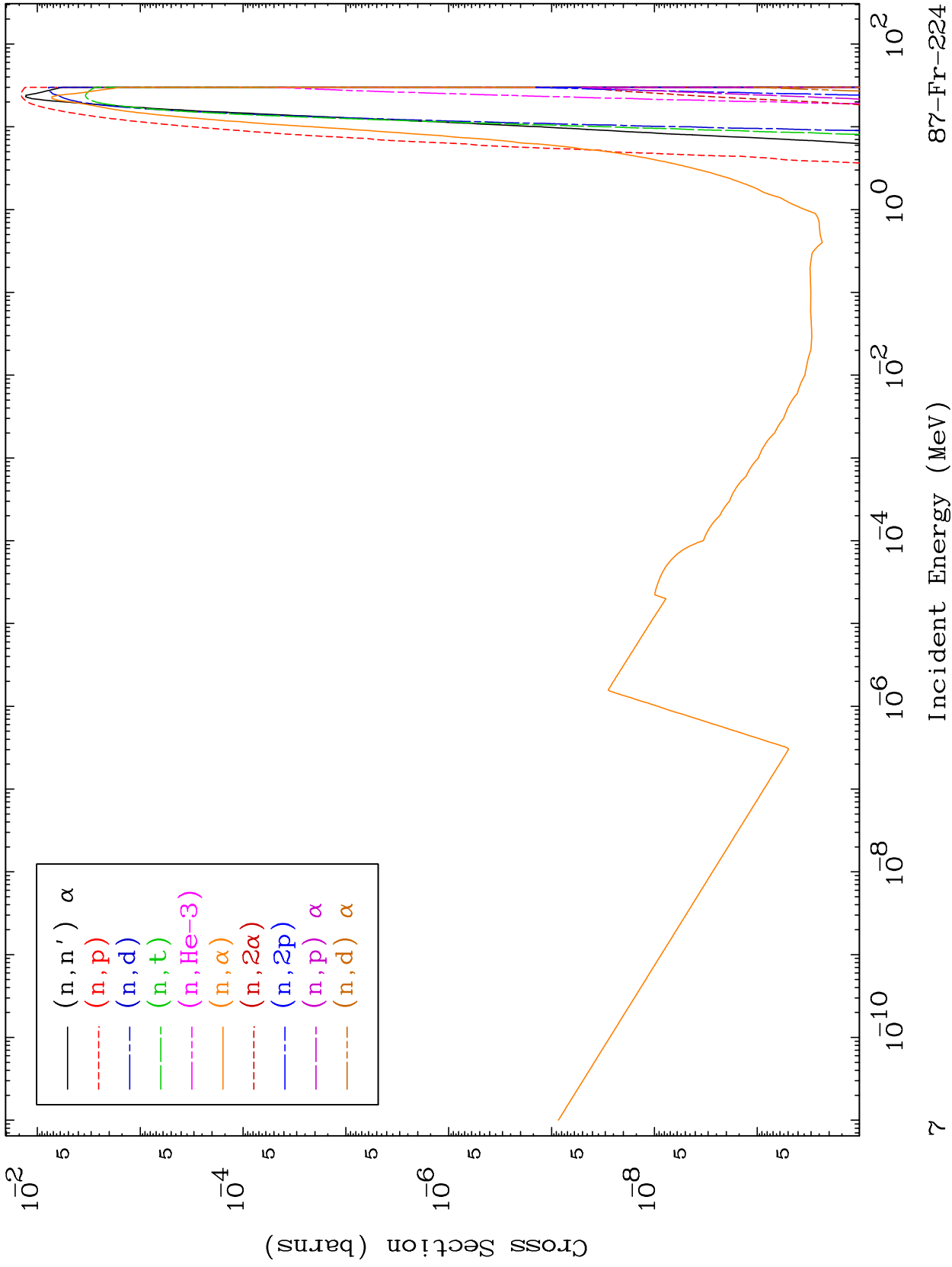
87-Fr-224

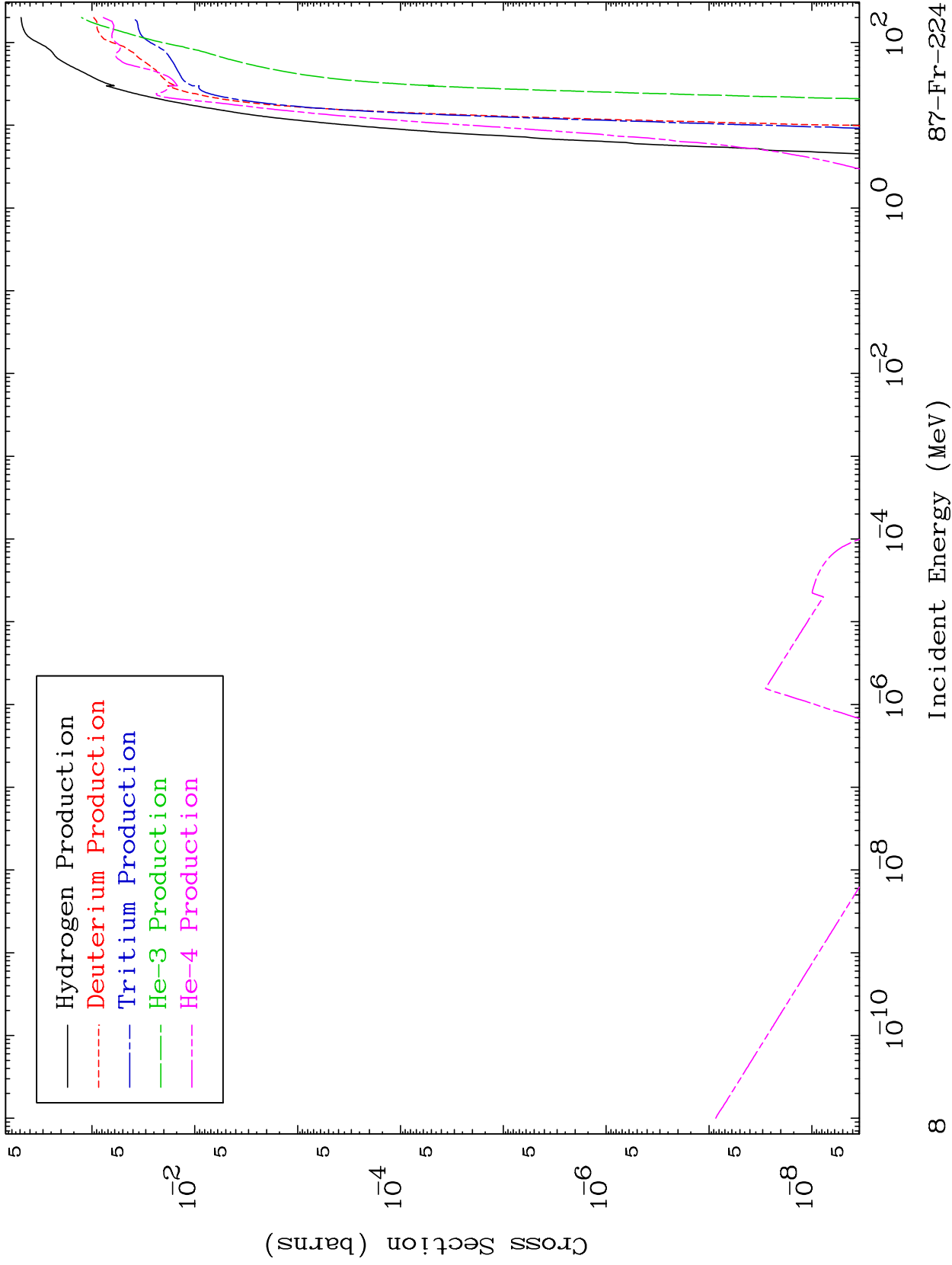


MAT 8761

Charged Particle
293 Kelvin Cross Sections

87-Fr-224

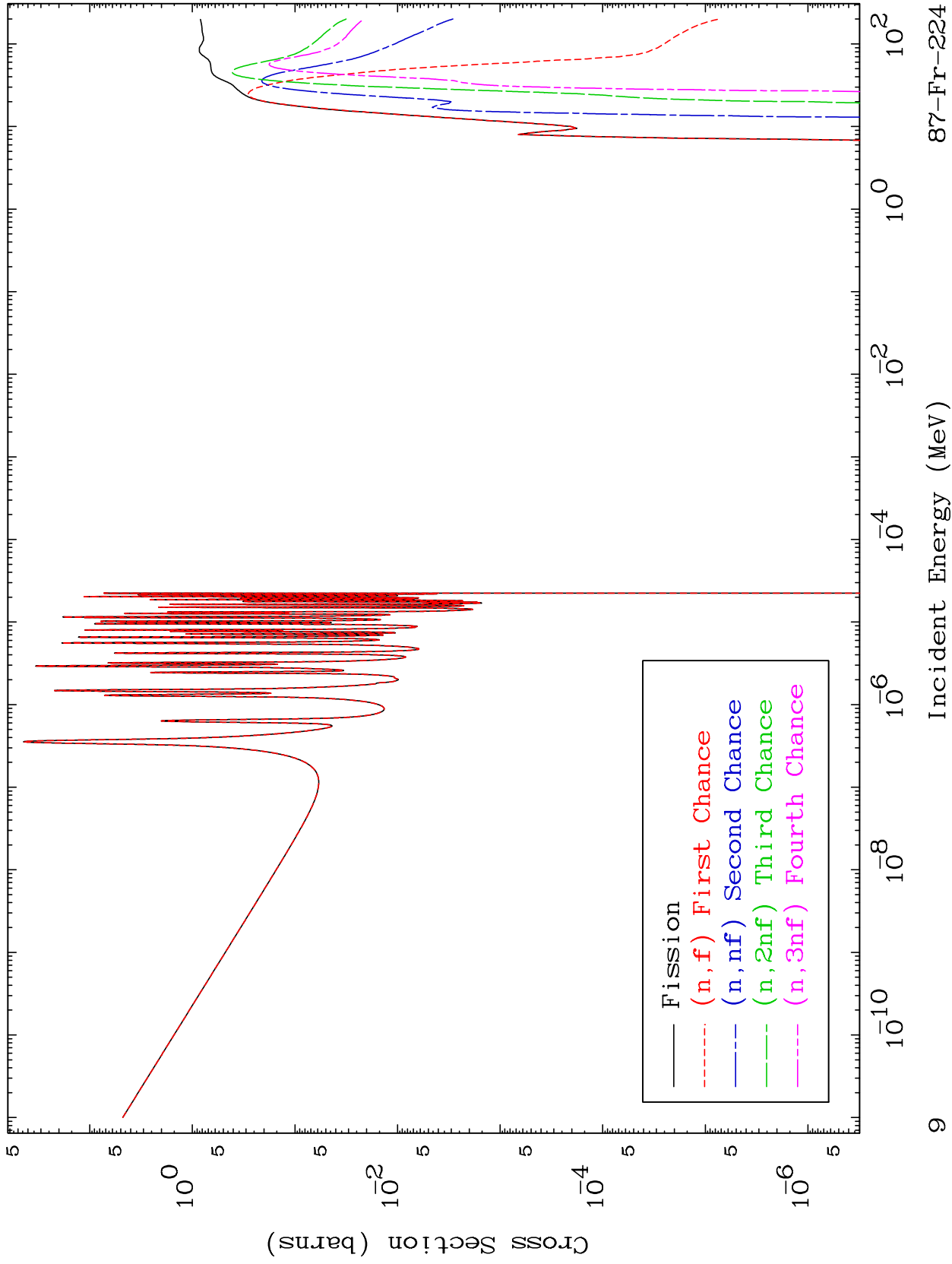




MAT 8761

Fission
293 Kelvin Cross Sections

87-Fr-224

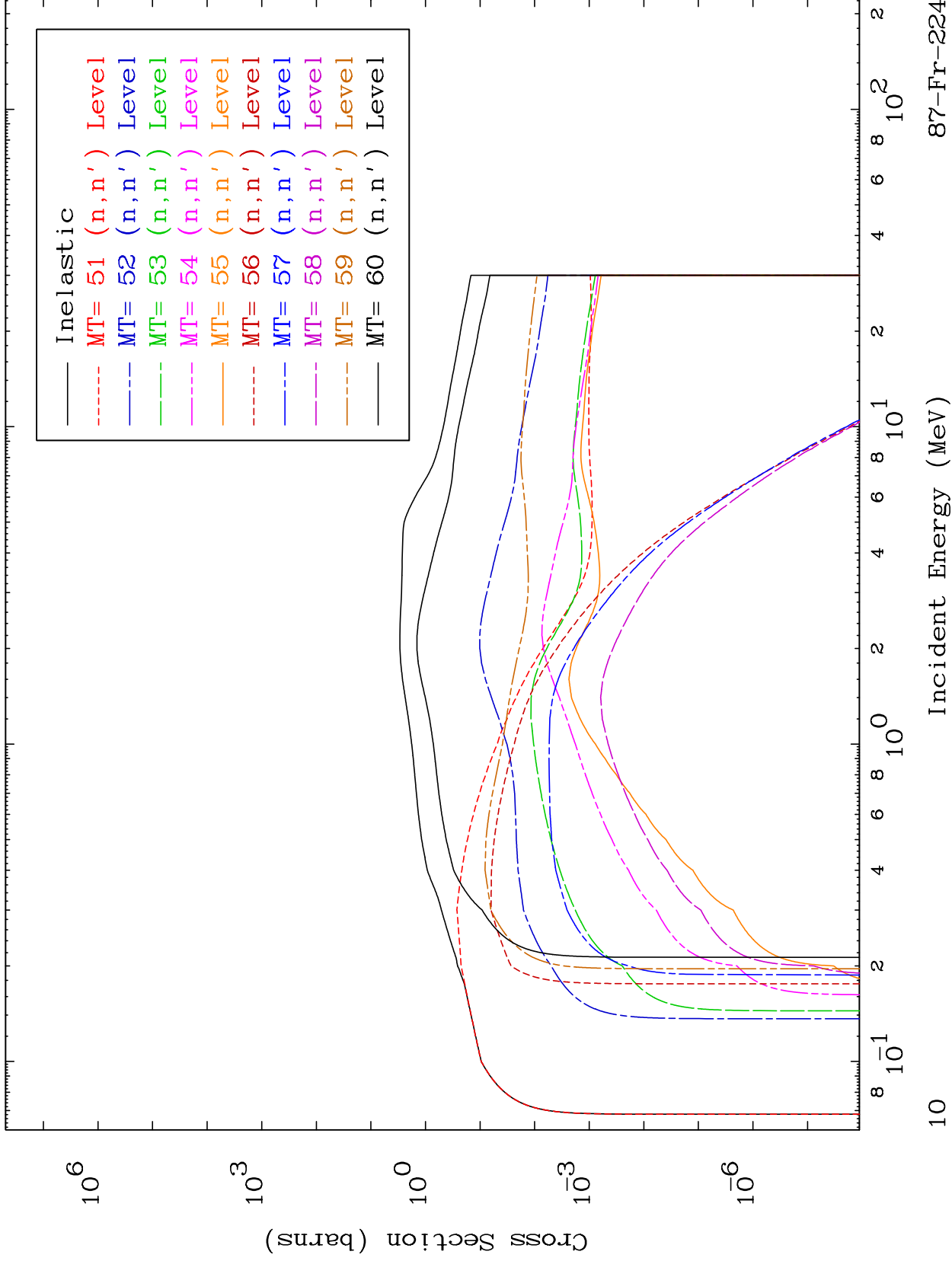


MAT 8761

(n,n') Levels

87-Fr-224

293 Kelvin Cross Sections

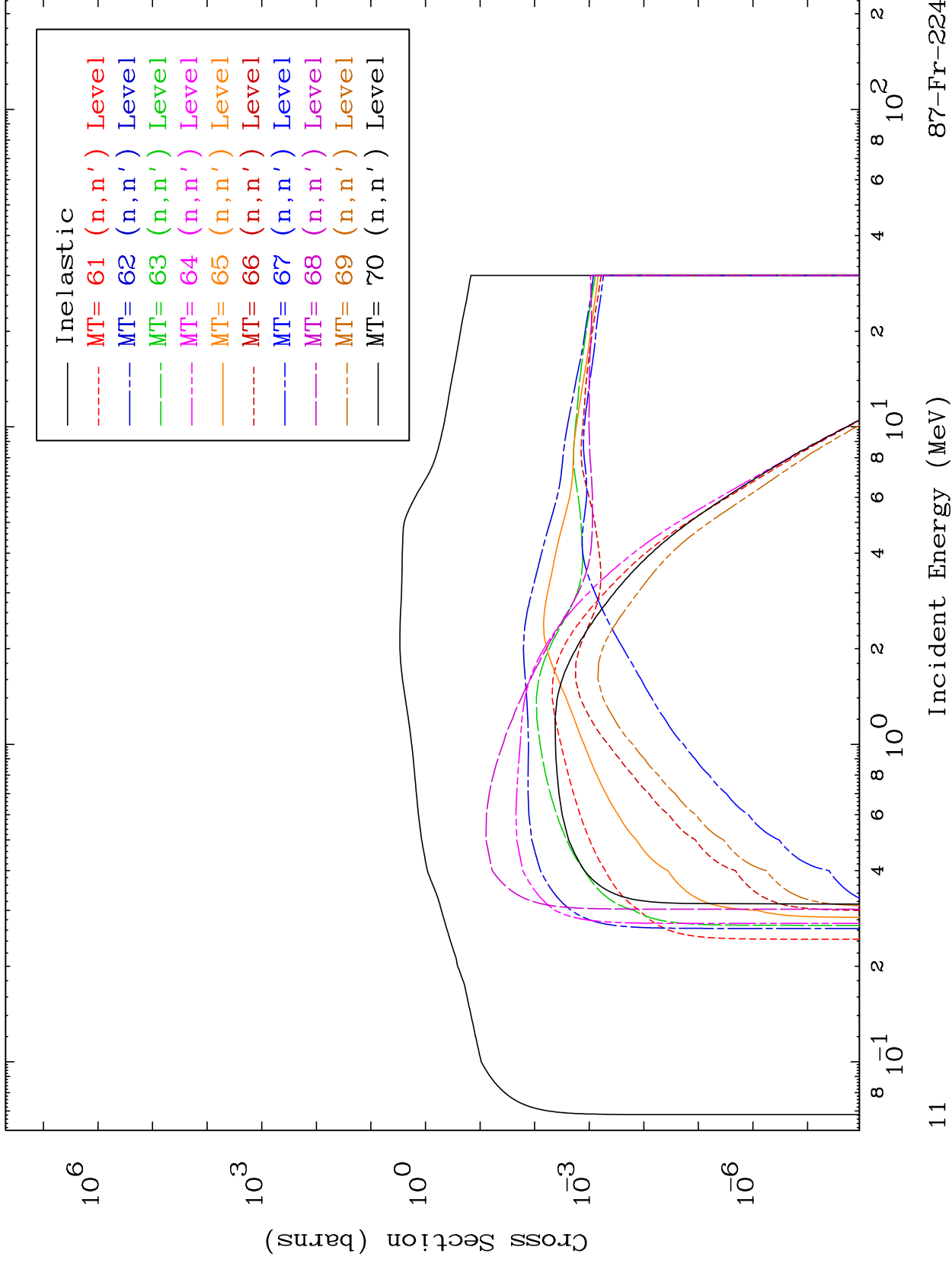


10

Incident Energy (MeV)

87-Fr-224

293 Kelvin Cross Sections

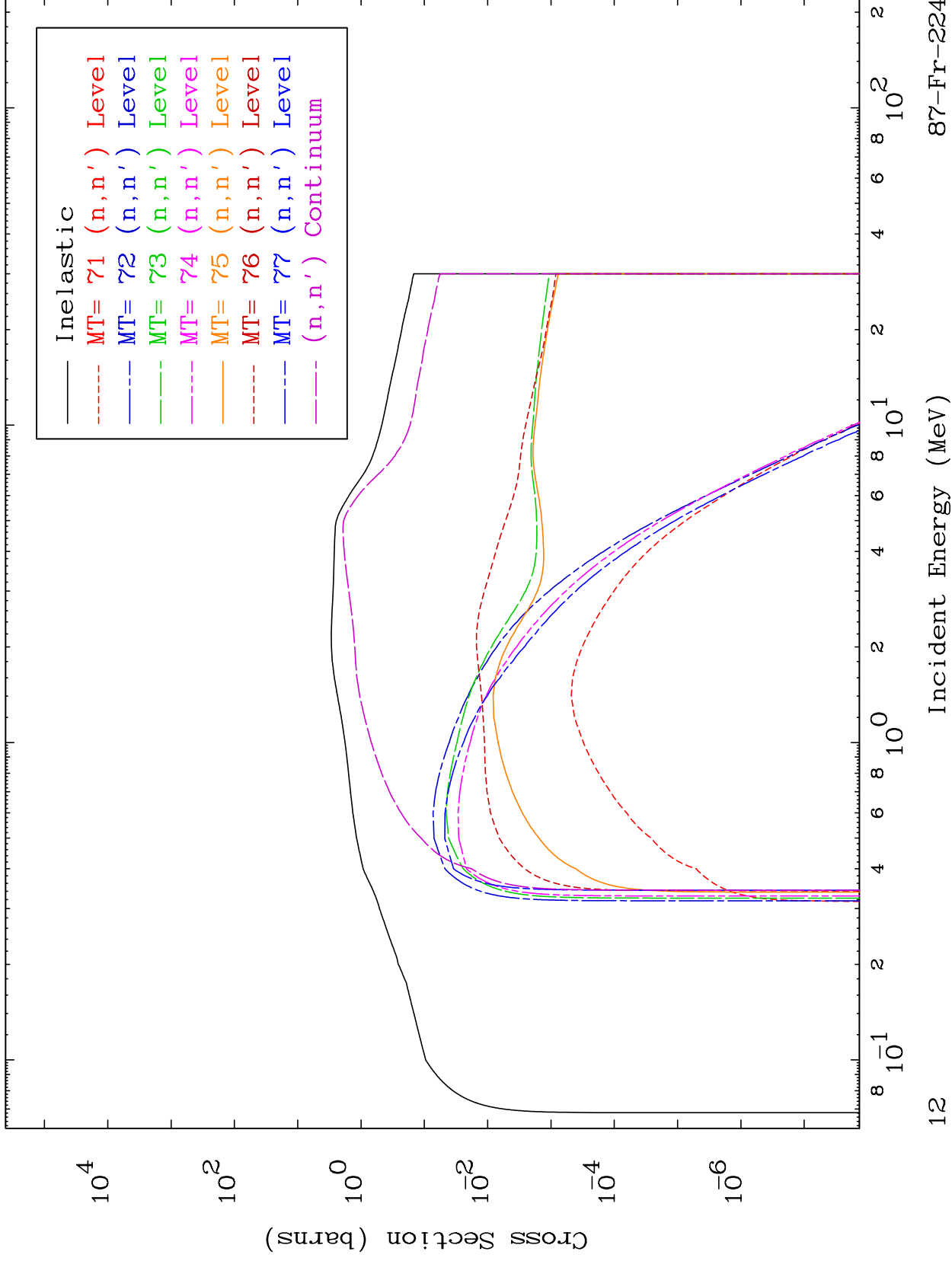


MAT 8761

(n,n') Levels

293 Kelvin Cross Sections

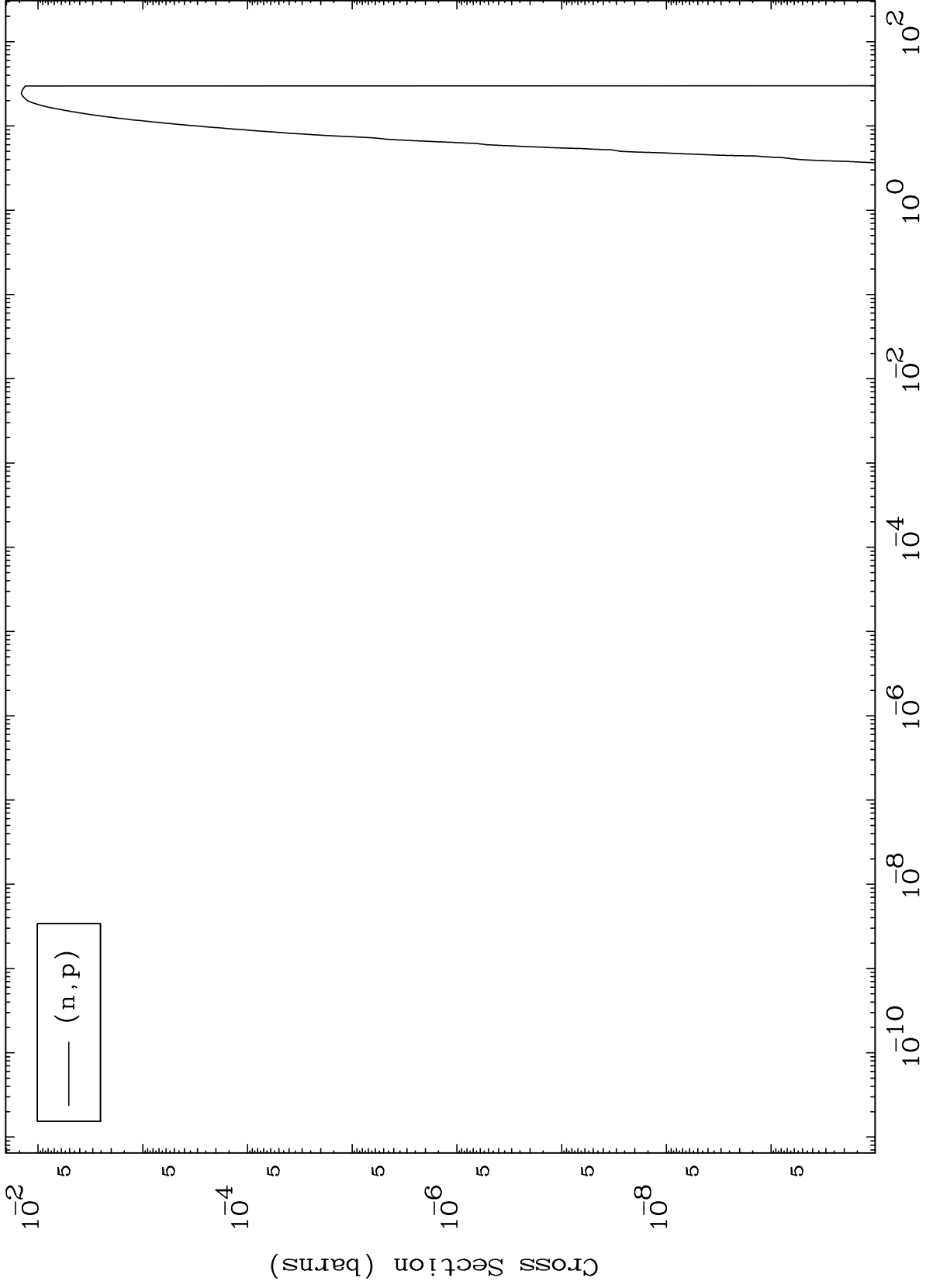
87-Fr-224



MAT 8761

(n,p) Levels
293 Kelvin Cross Sections

87-Fr-224



13

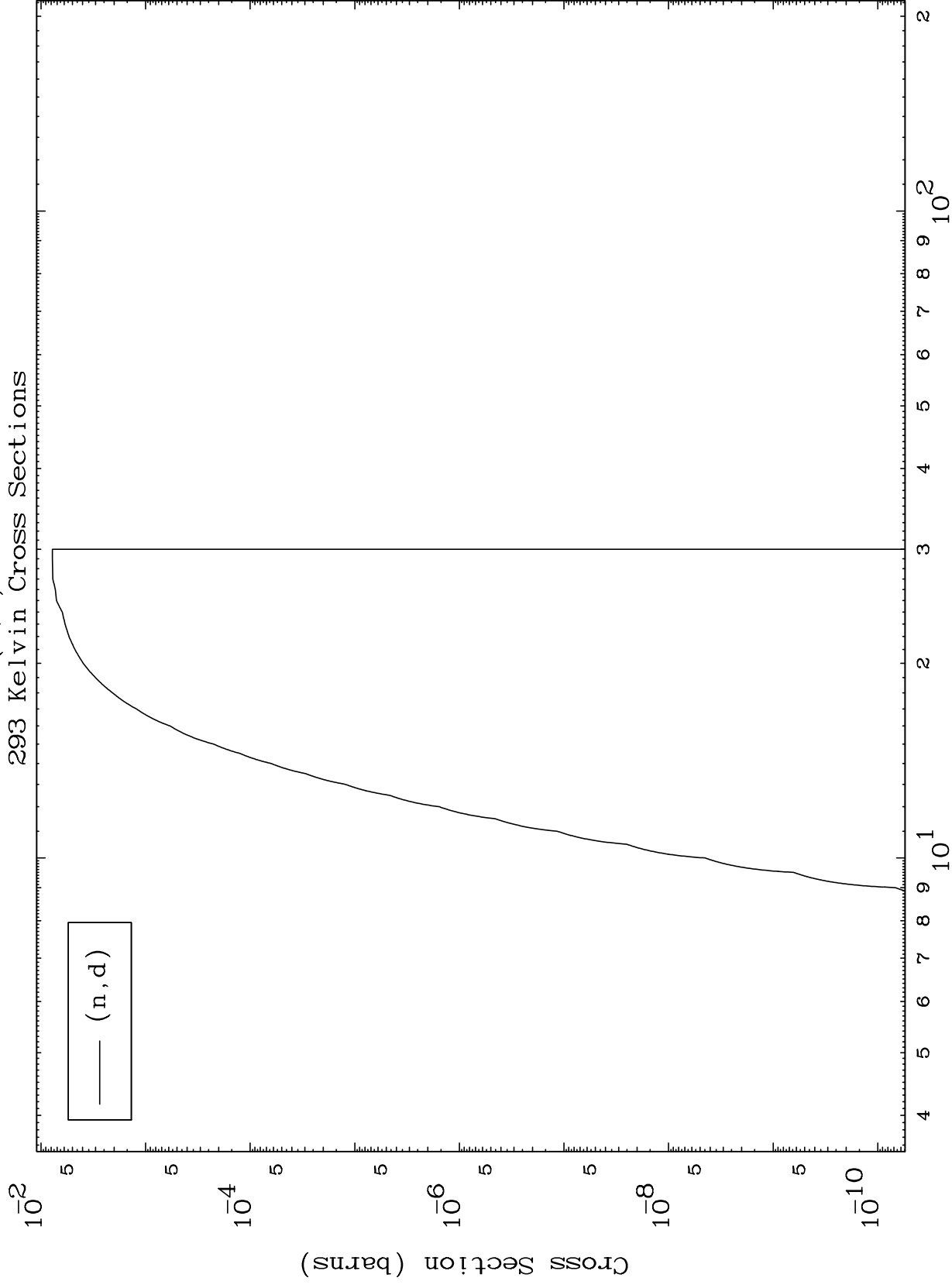
Incident Energy (MeV)

87-Fr-224

MAT 8761

(n,d) Levels
293 Kelvin Cross Sections

87-Fr-224



14

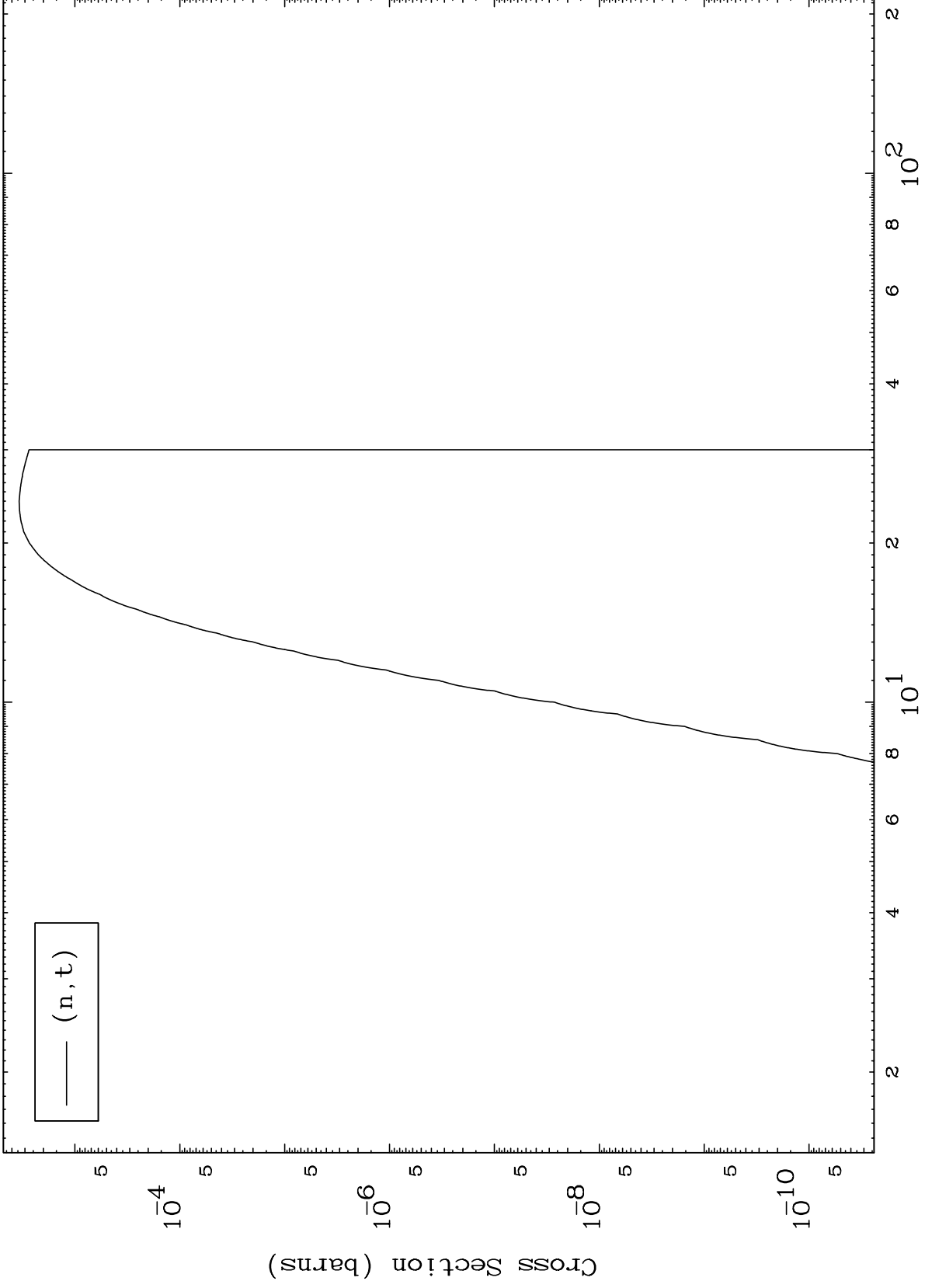
Incident Energy (MeV)

87-Fr-224

MAT 8761

(n,t) Levels
293 Kelvin Cross Sections

87-Fr-224



15

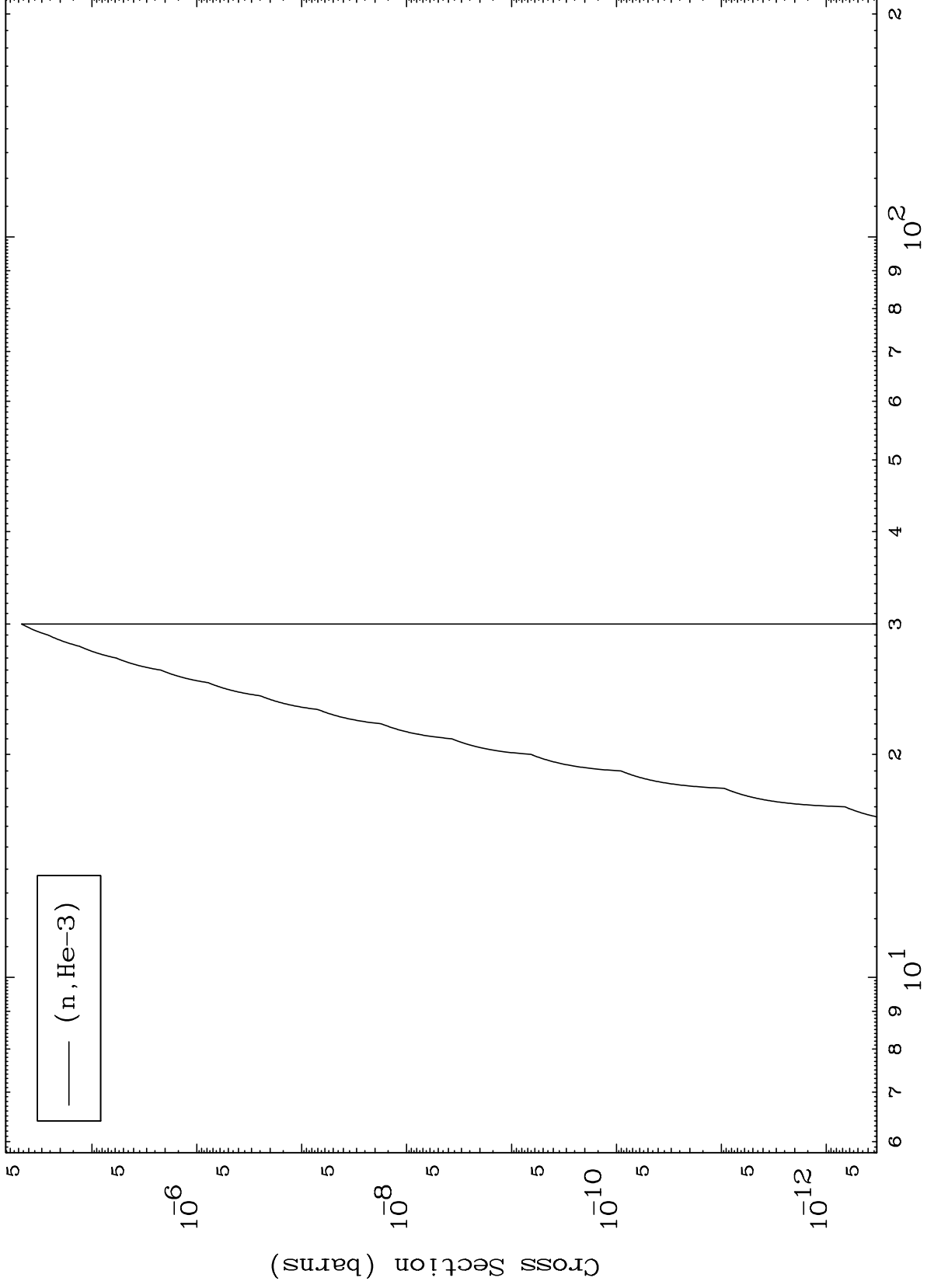
Incident Energy (MeV)

87-Fr-224

MAT 8761

(n,He3) Levels
293 Kelvin Cross Sections

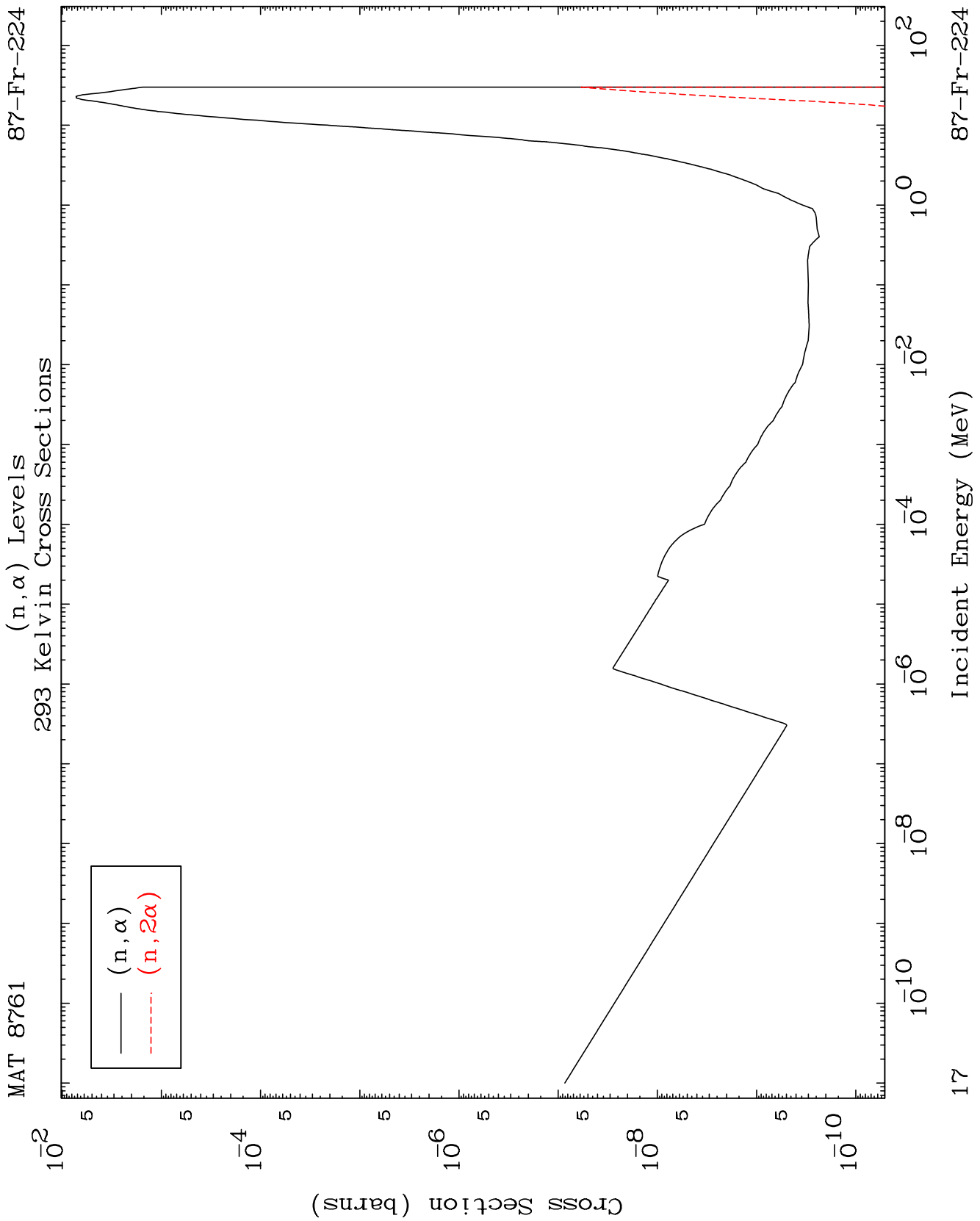
87-Fr-224



16

Incident Energy (MeV)

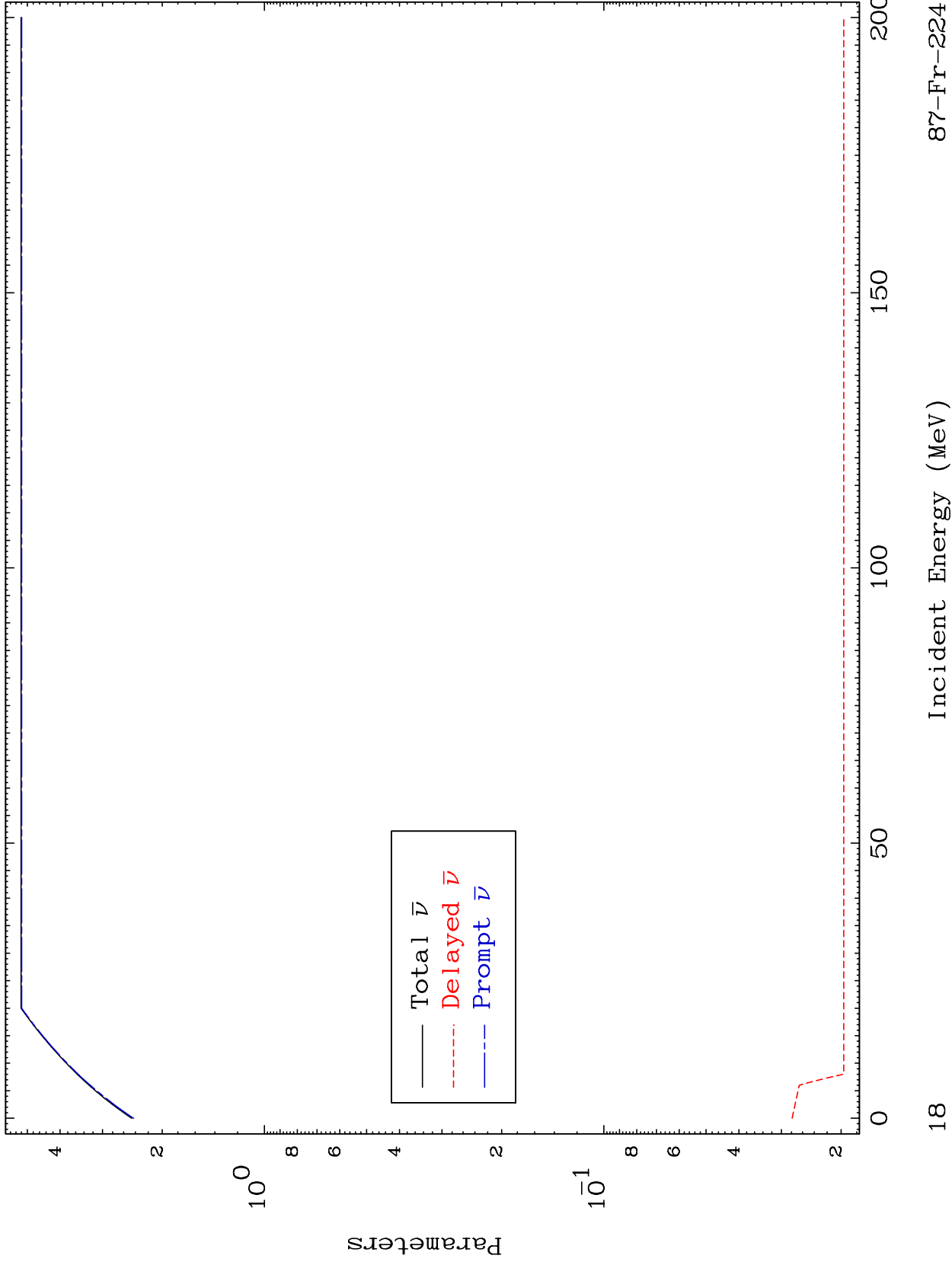
87-Fr-224



MAT 8761

Energy Release
Parameters

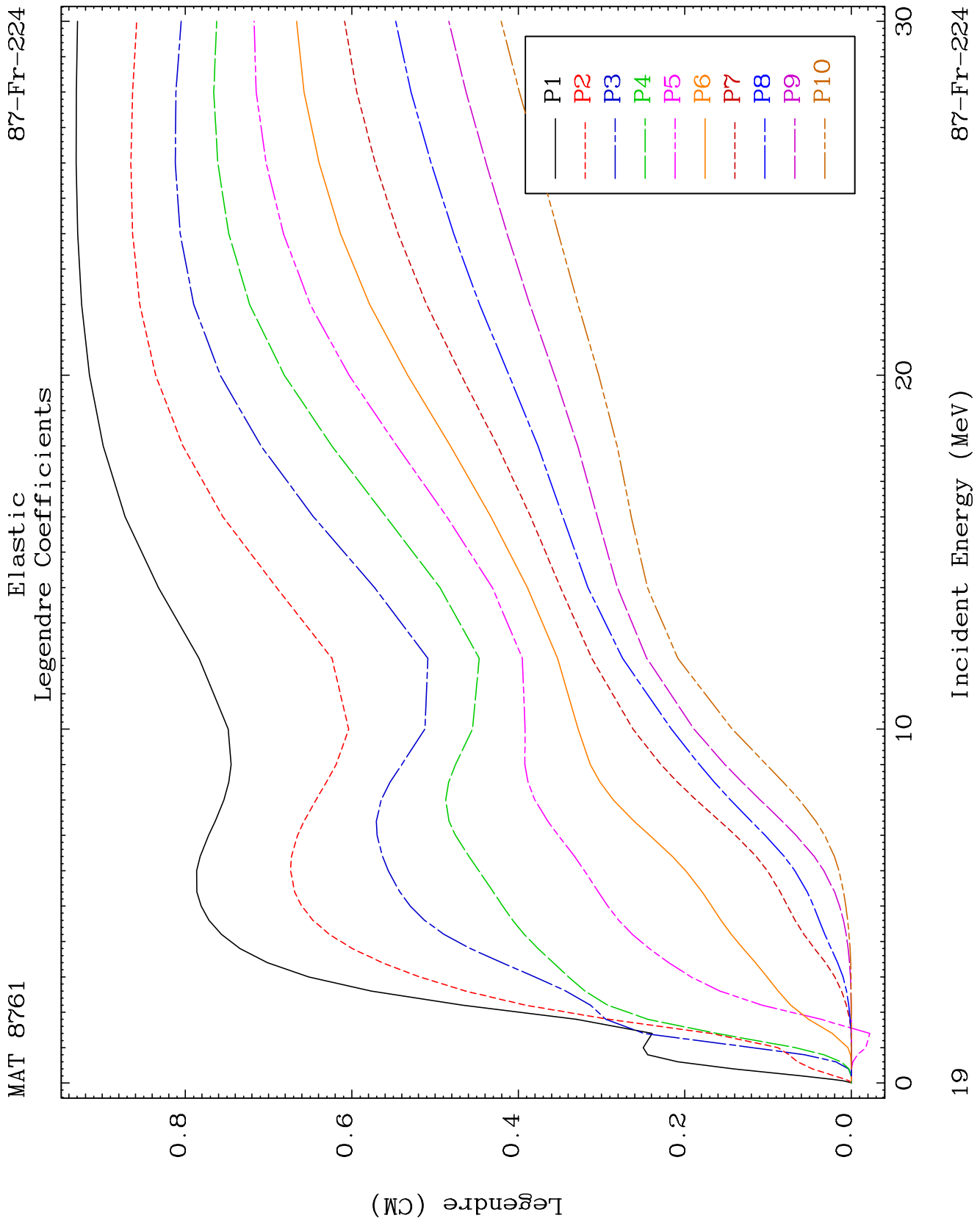
87-Fr-224

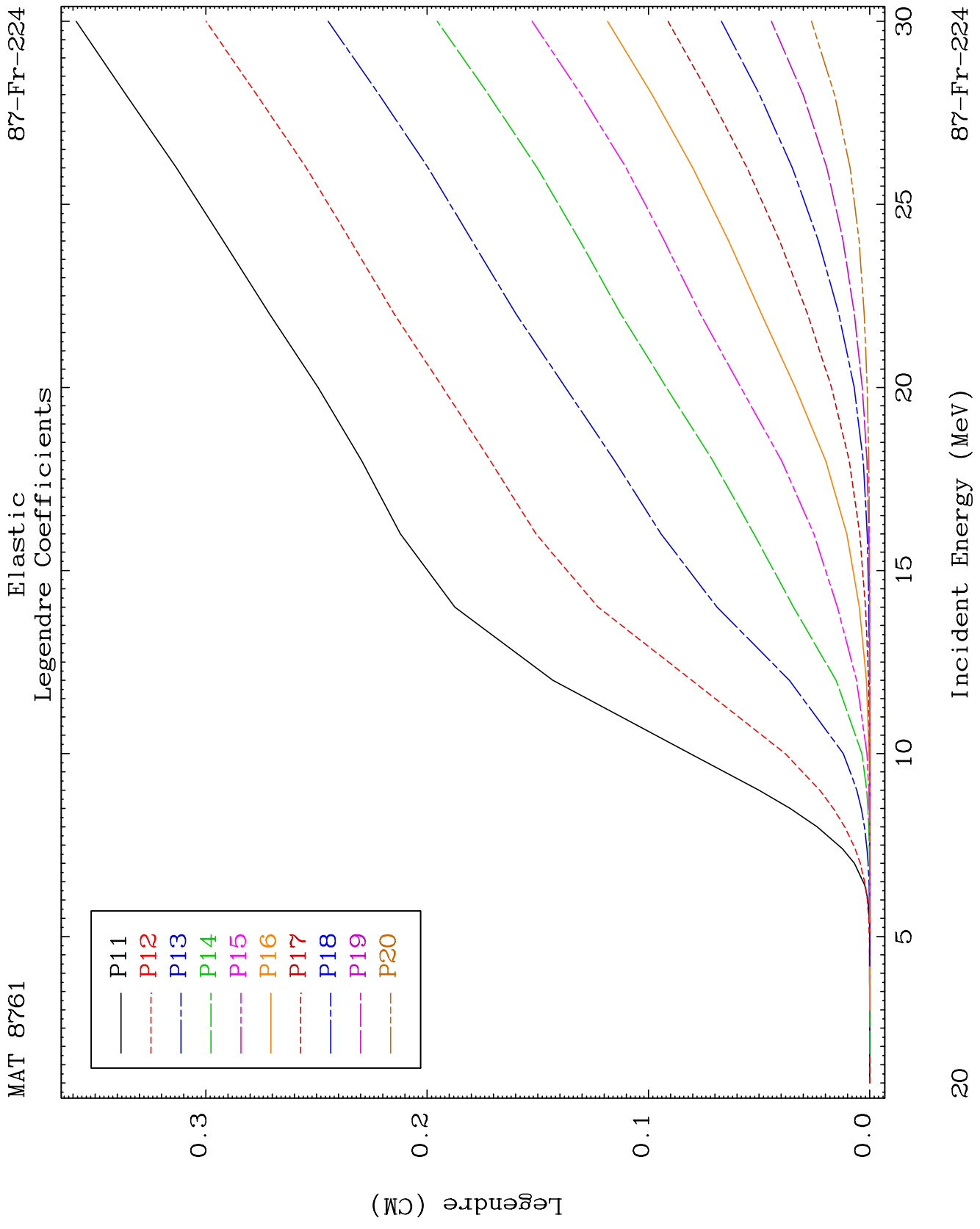


18

Incident Energy (MeV)

87-Fr-224

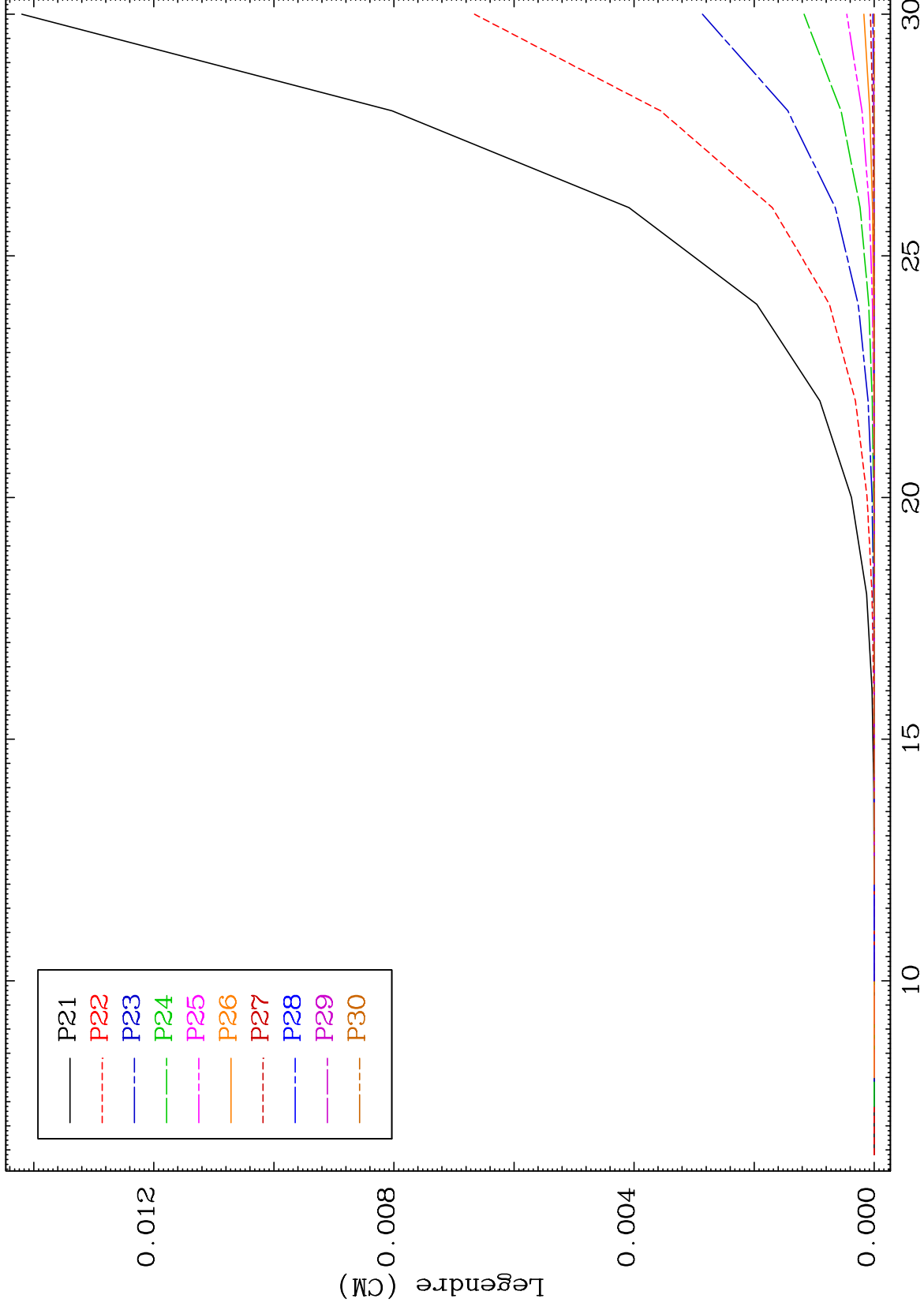




MAT 8761

Elastic
Legendre Coefficients

87-Fr-224



21

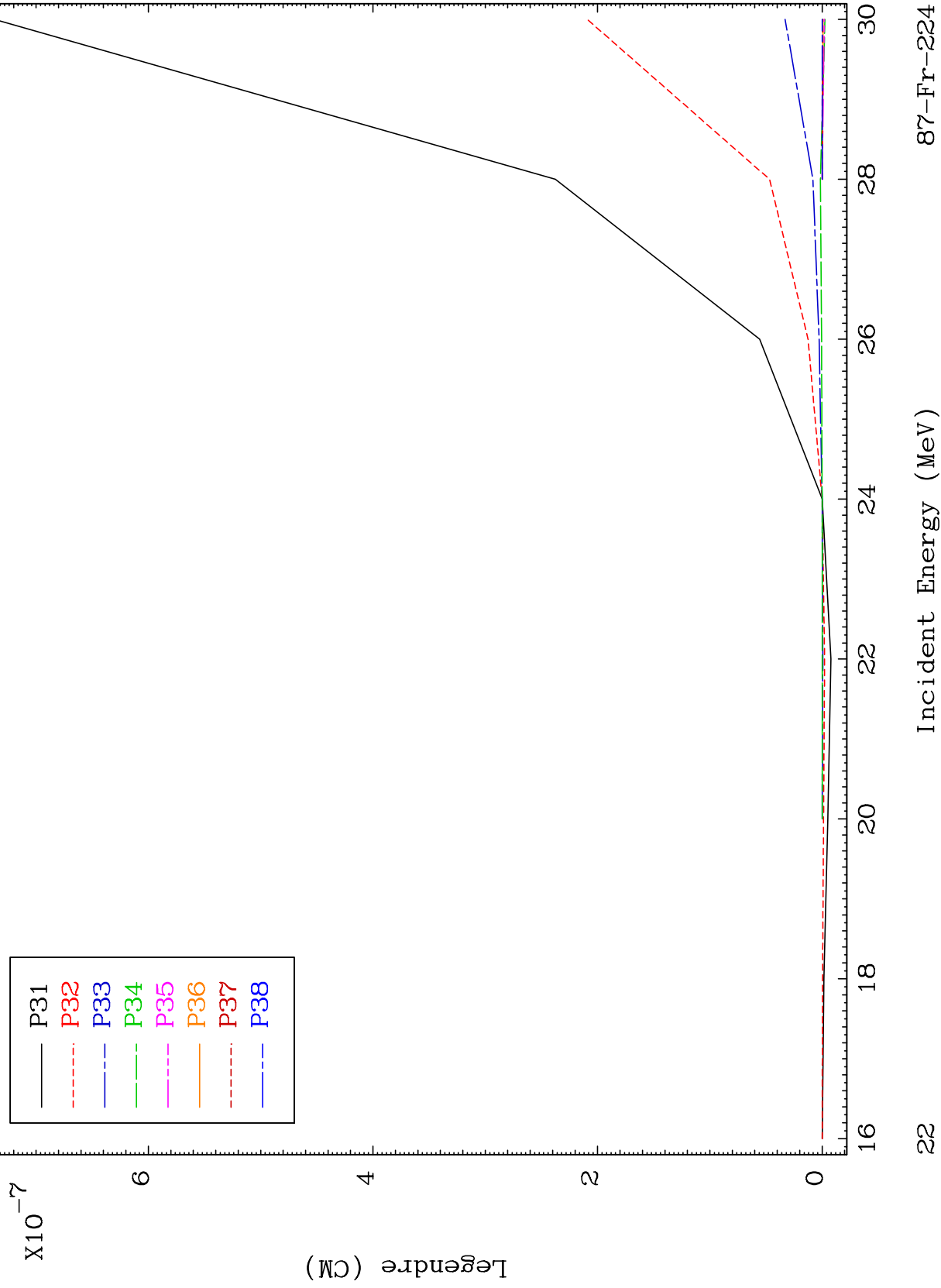
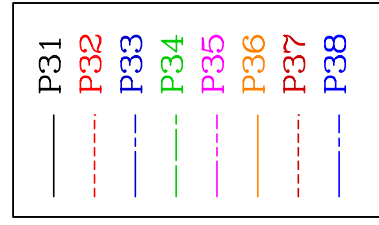
Incident Energy (MeV)

87-Fr-224

MAT 8761

Elastic Legendre Coefficients

87-Fr-224



87-Fr-224

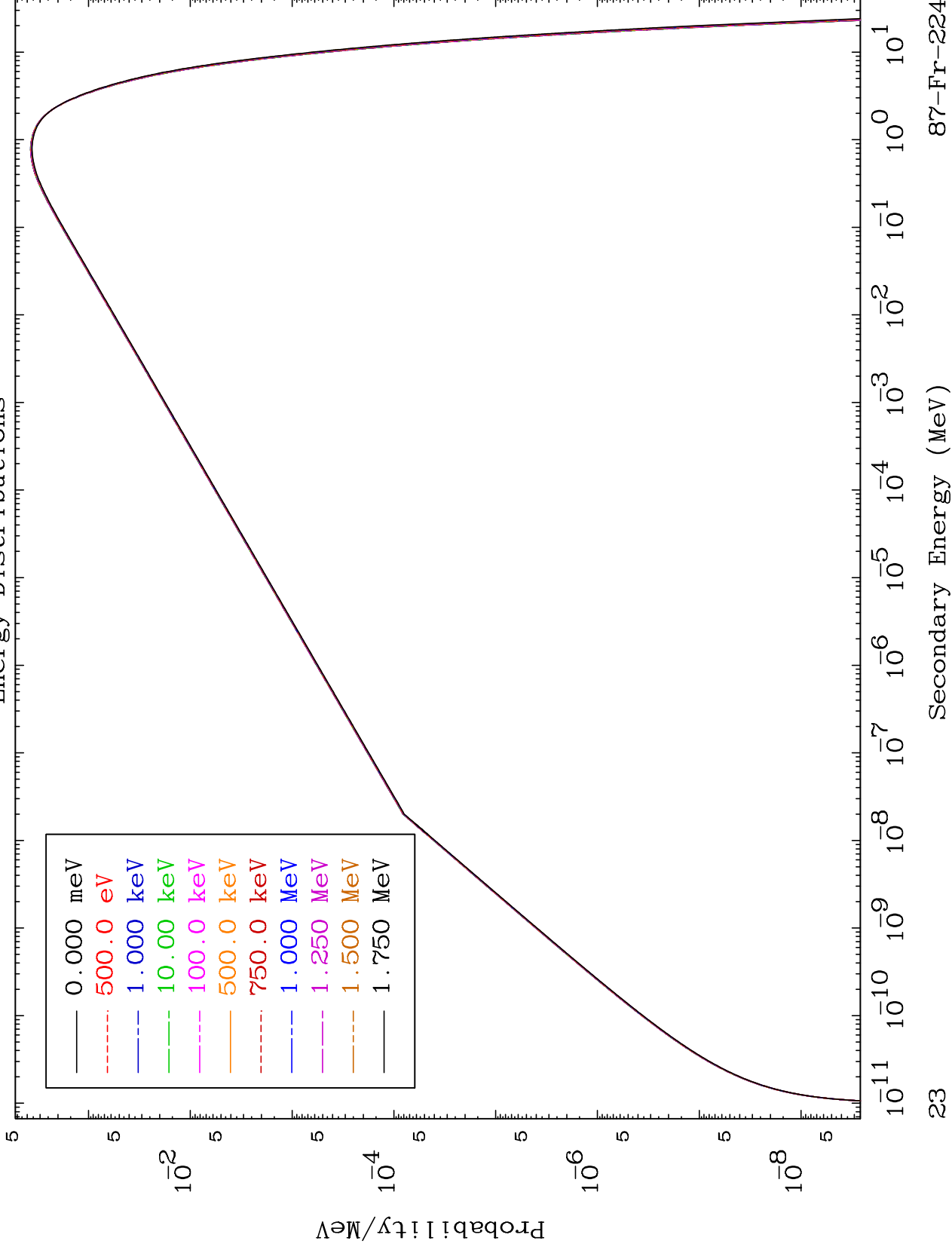
Incident Energy (MeV)

22

MAT 8761

Fission Energy Distributions

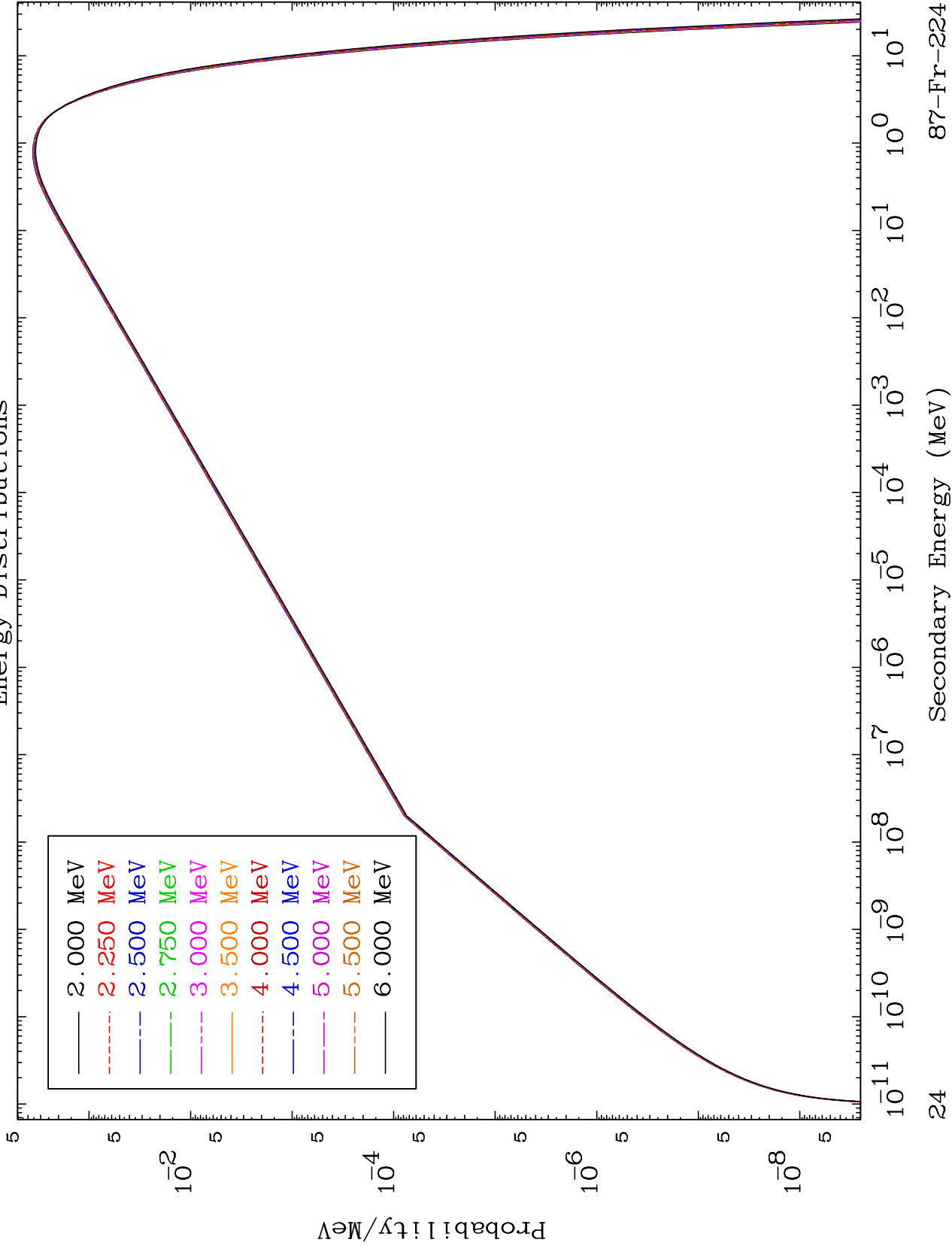
87-Fr-224



MAT 8761

Fission Energy Distributions

87-Fr-224

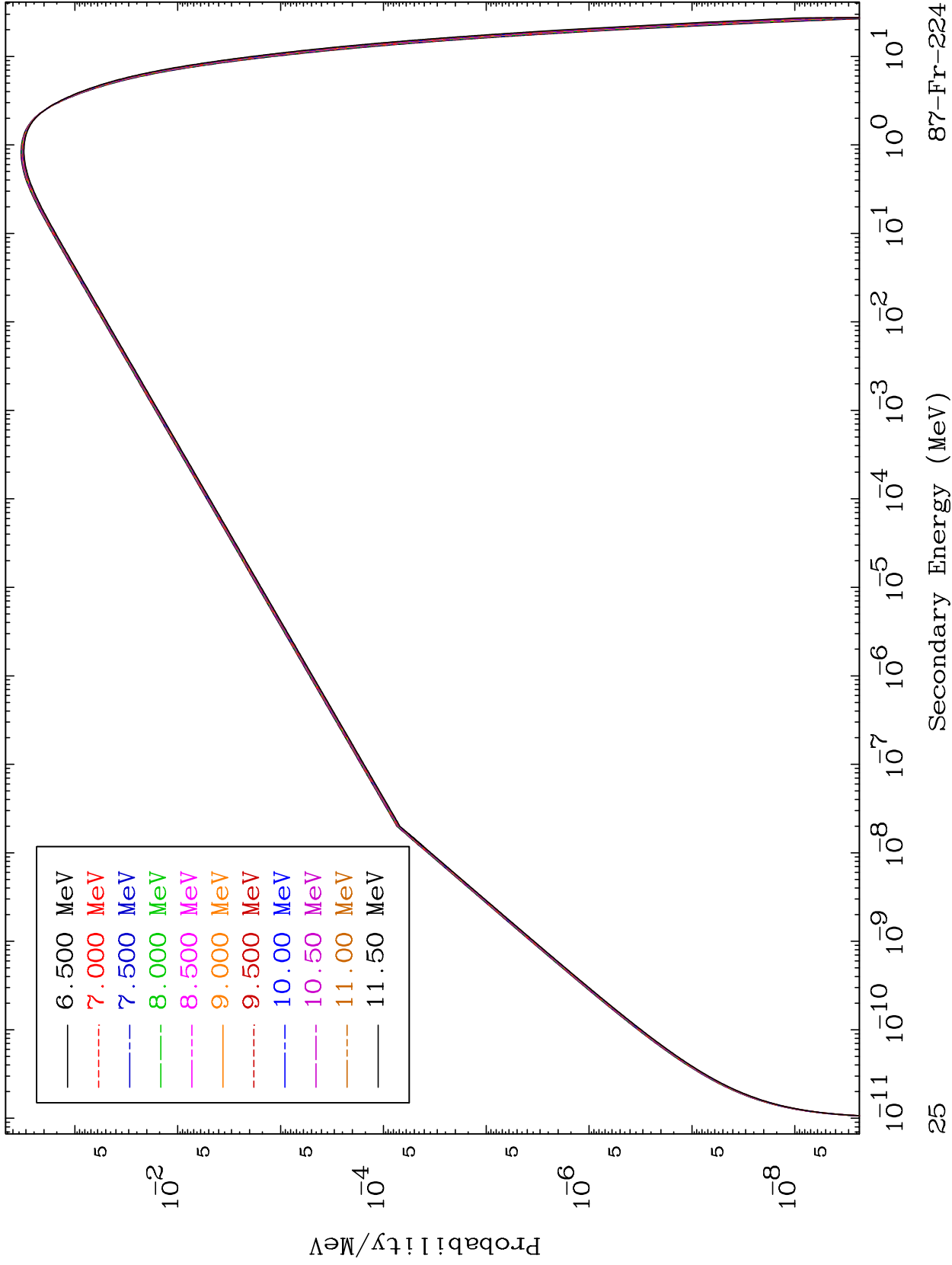


24

MAT 8761

Fission Energy Distributions

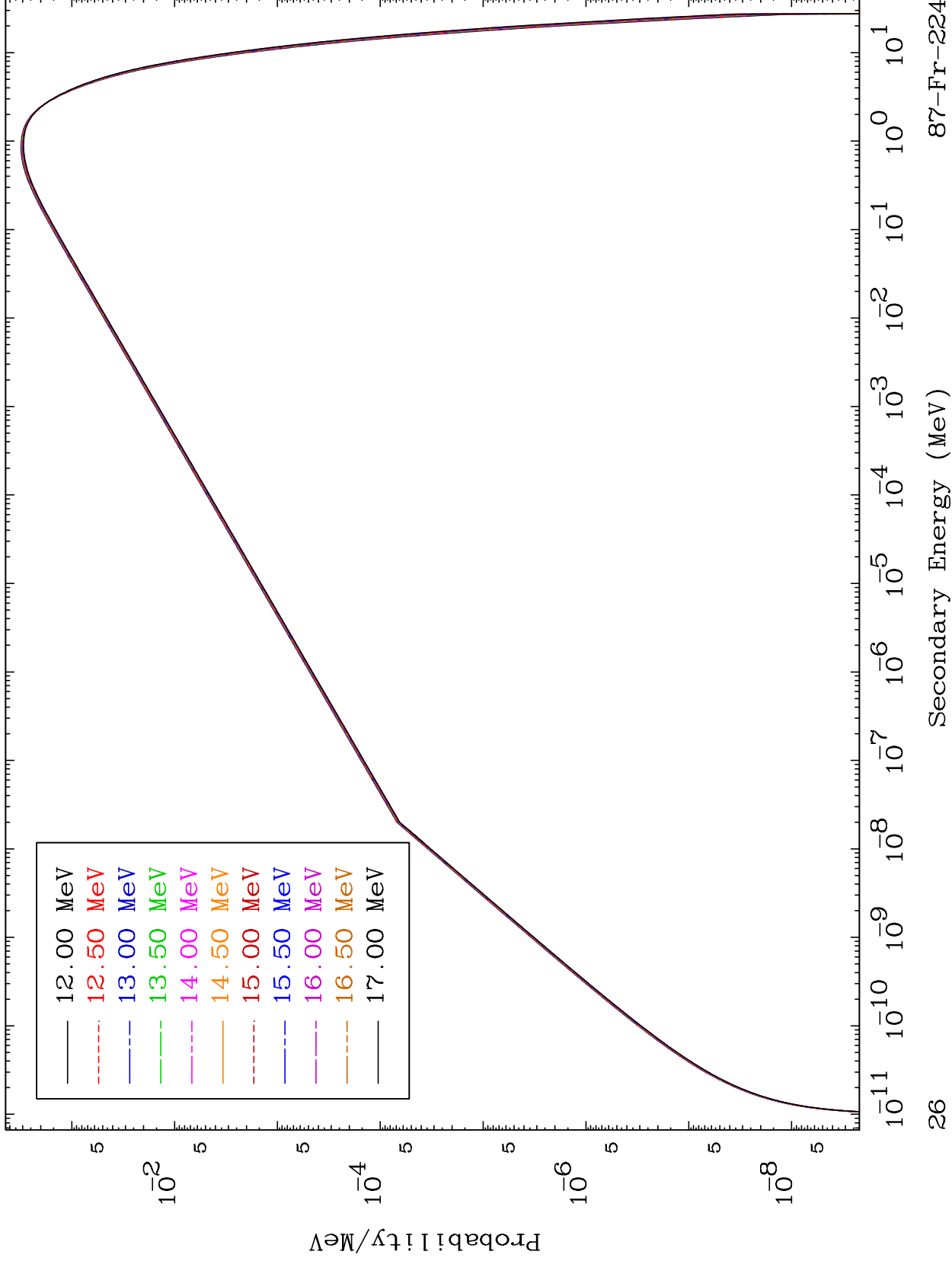
87-Fr-224



MAT 8761

Fission Energy Distributions

87-Fr-224



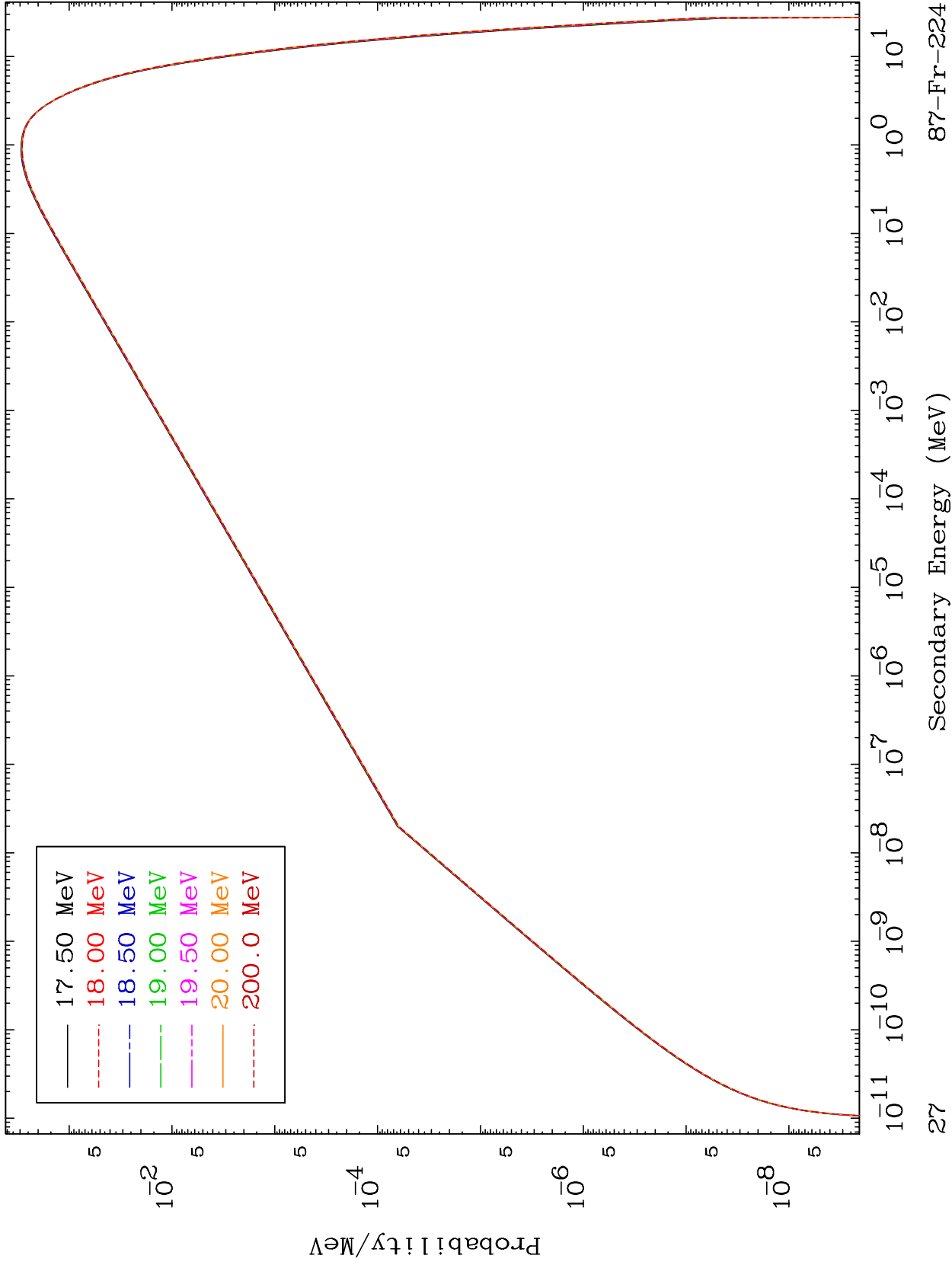
26

87-Fr-224

MAT 8761

Fission Energy Distributions

⁸⁷Fr-224

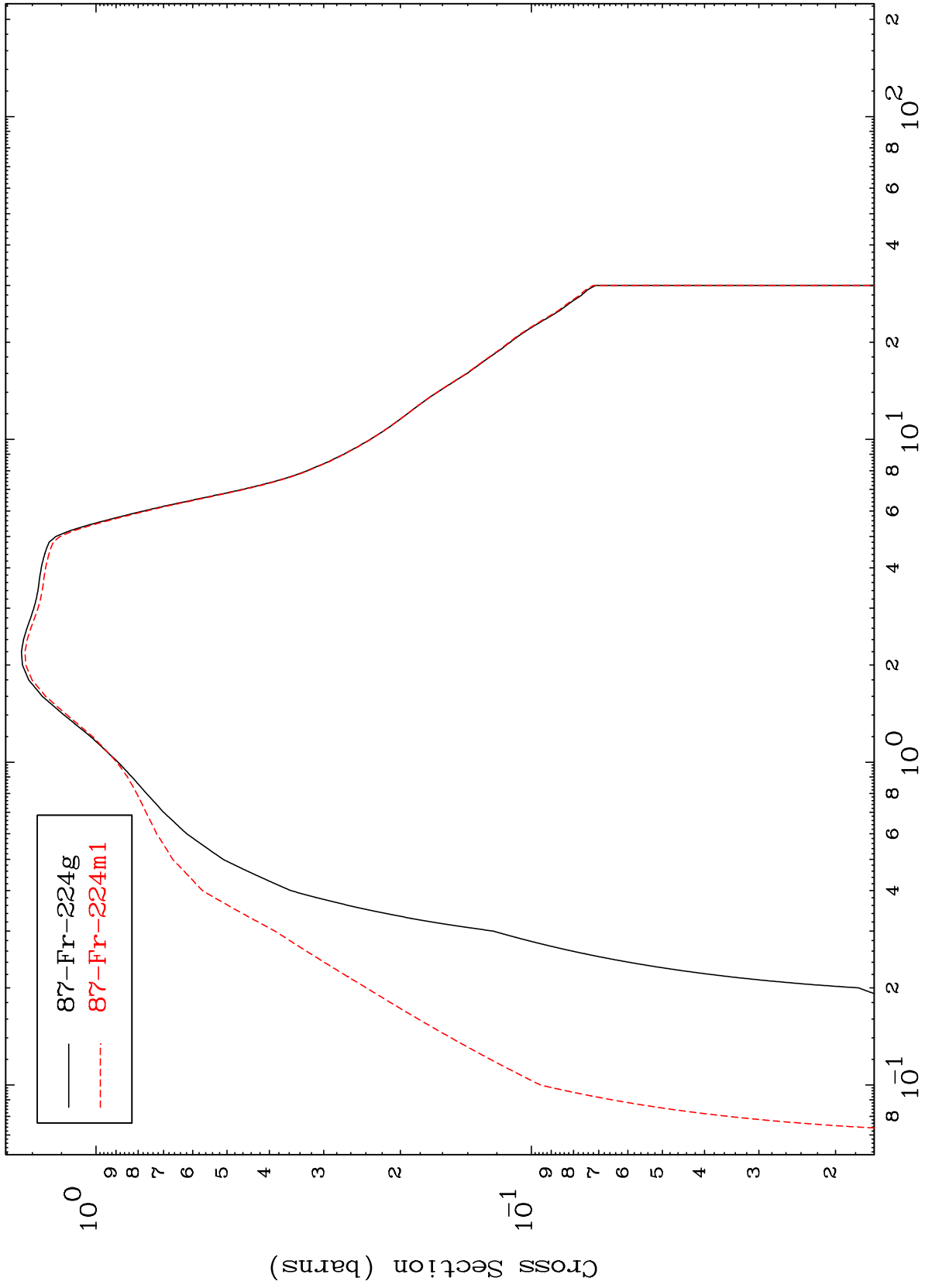


27

MAT 8761

87-Fr-224

Radionuclide Production Cross Section



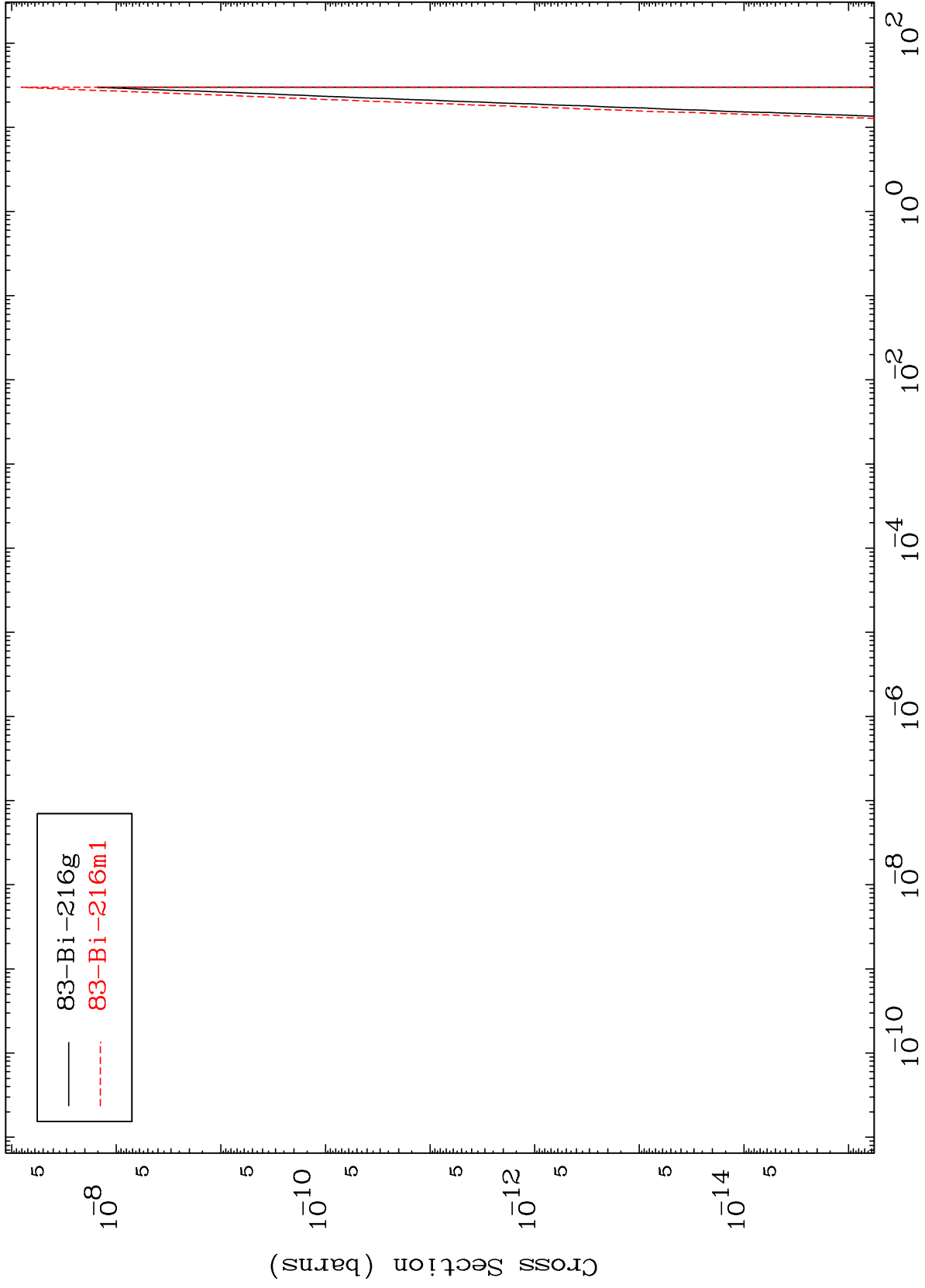
87-Fr-224g
87-Fr-224m1

MAT 8761

(n,n') 2 α

87-Fr-224

Radionuclide Production Cross Section



29

Incident Energy (MeV)

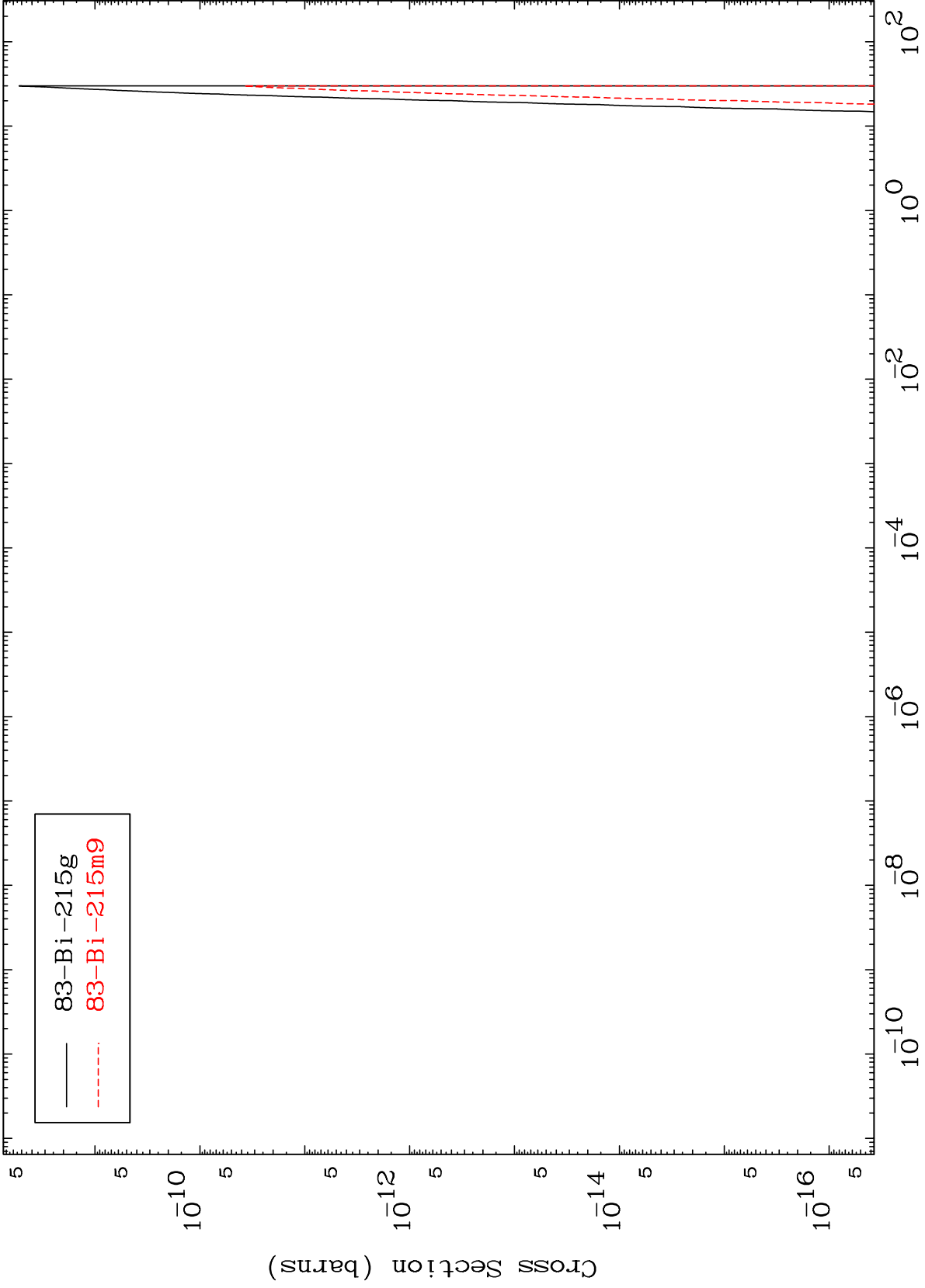
87-Fr-224

MAT 8761

(n,2n) 2 α

87-Fr-224

Radionuclide Production Cross Section



83-Bi-215g
83-Bi-215m9

30

Incident Energy (MeV)

87-Fr-224