

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

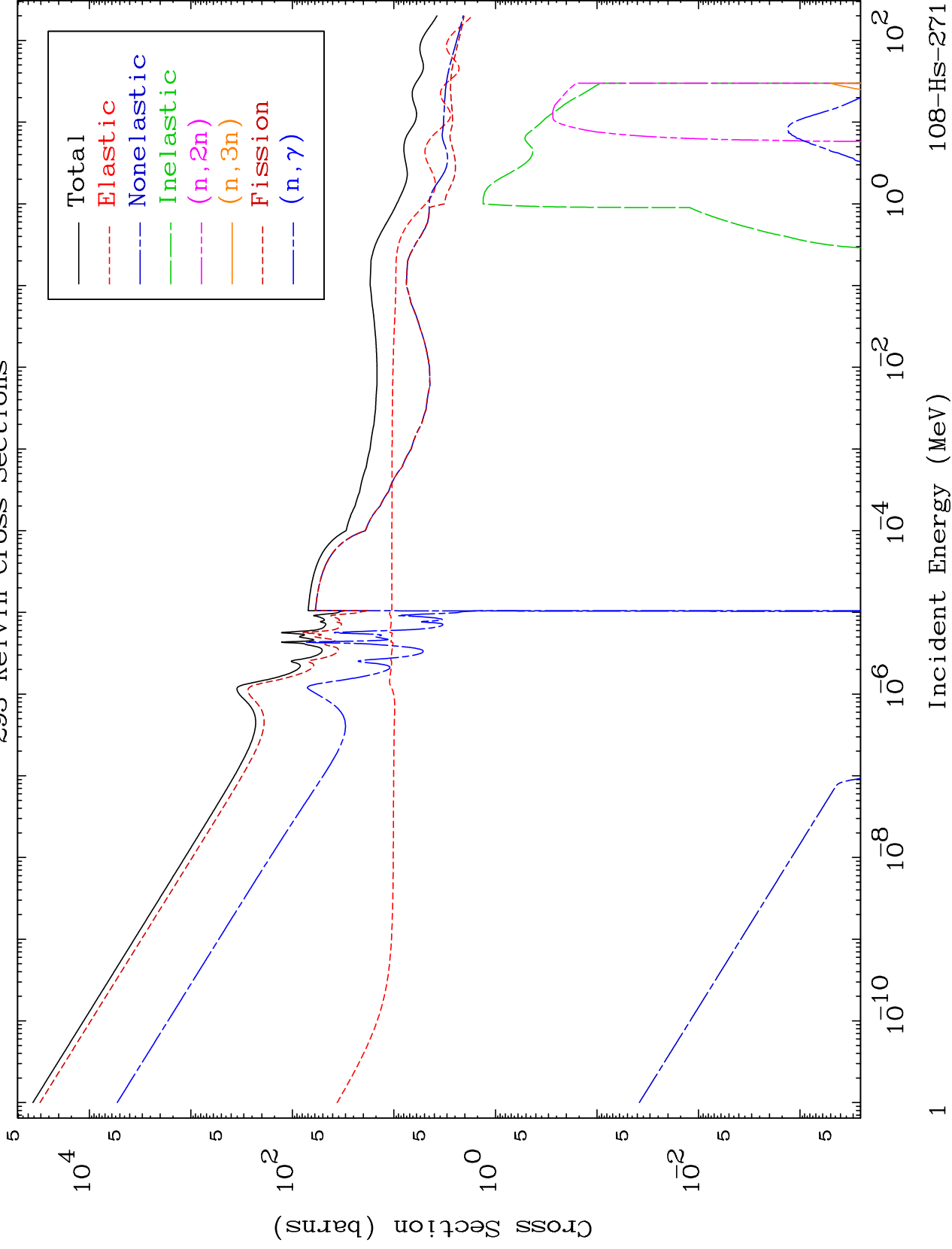
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Press Mouse Button to Start

MAT 871

Neutron Major  
293 Kelvin Cross Sections

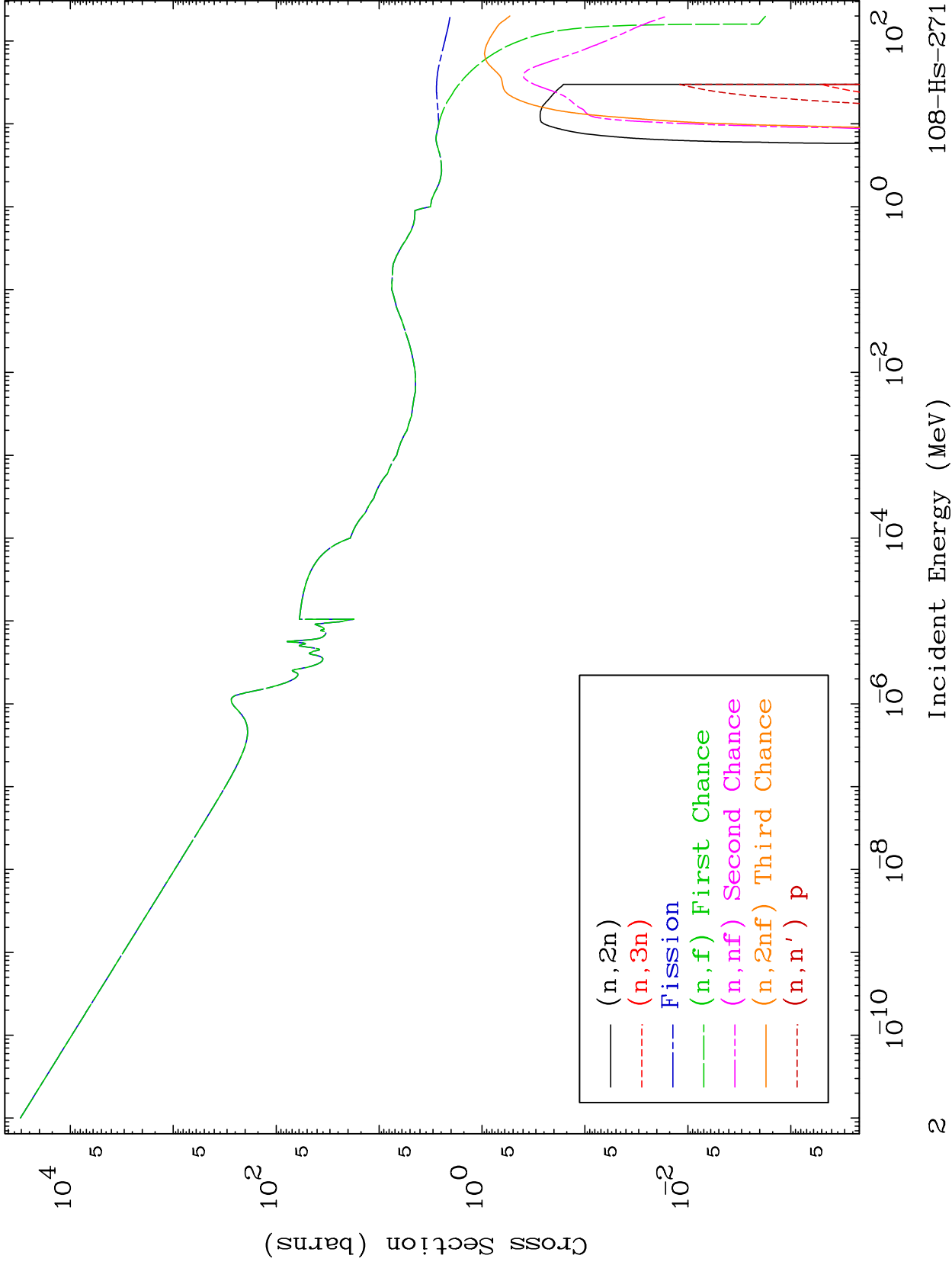
108-Hs-271

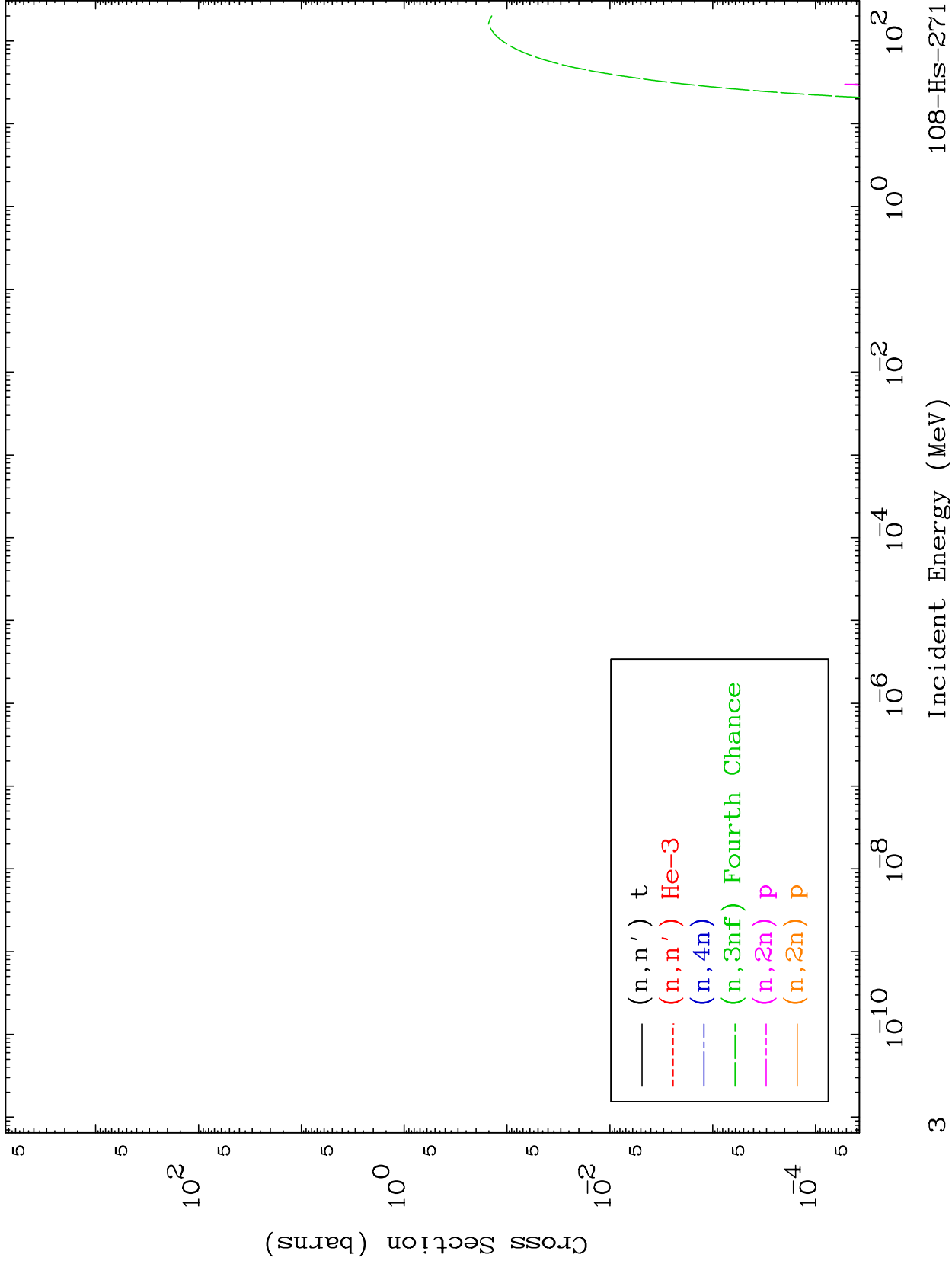


MAT 871

Neutron Absorption  
293 Kelvin Cross Sections

108-Hs-271

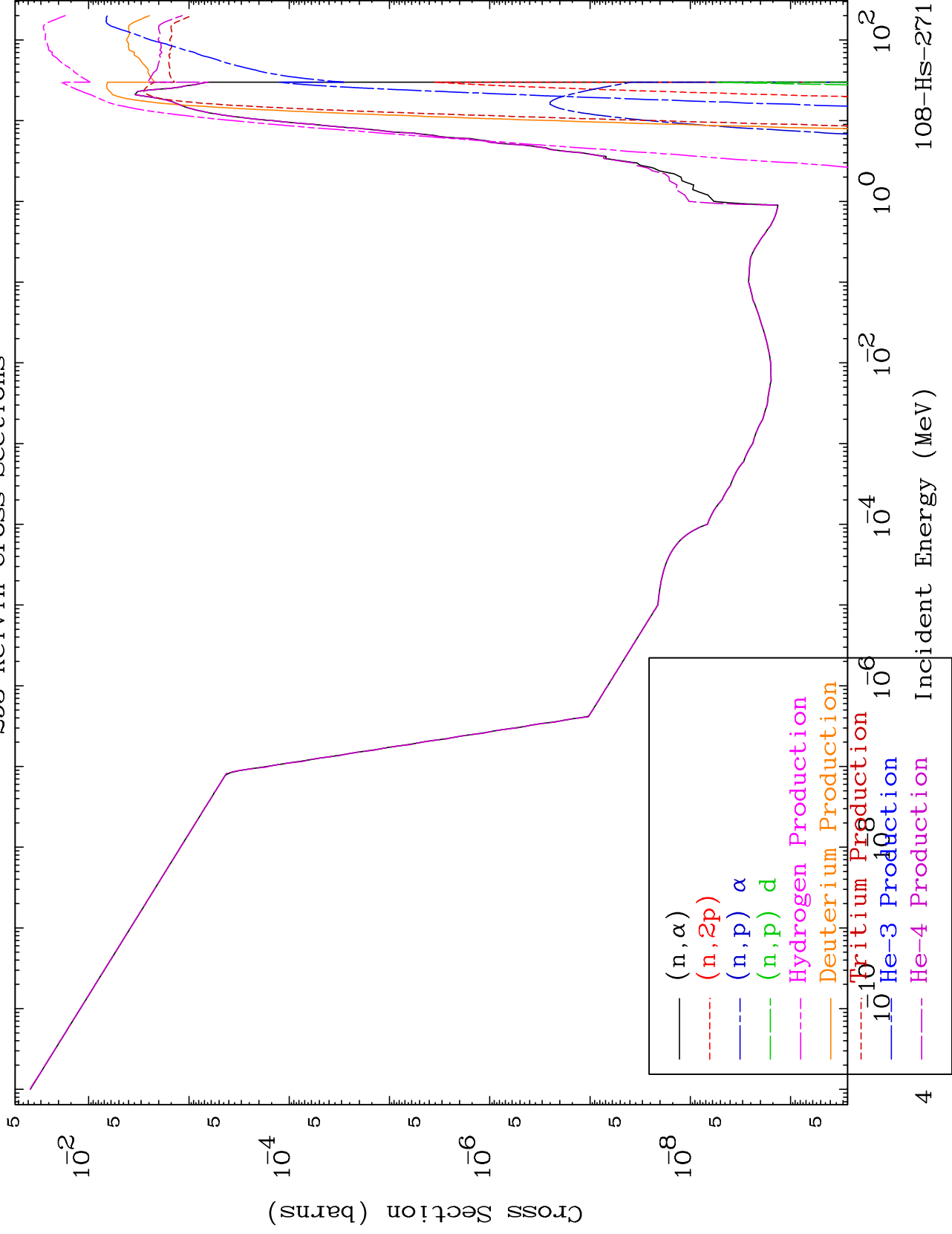




MAT 871

Neutron Absorption  
293 Kelvin Cross Sections

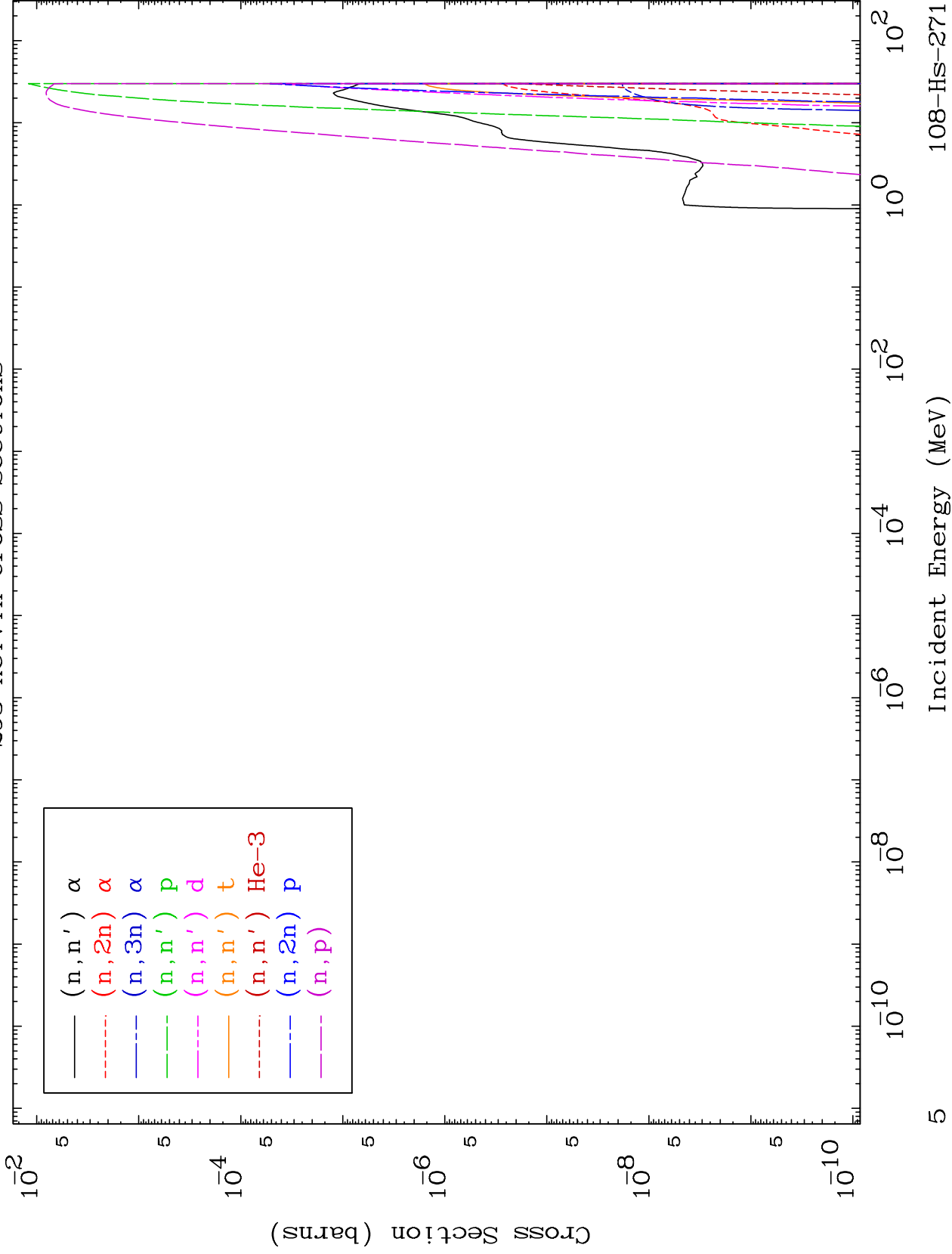
108-Hs-271



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Charged Particle  
293 Kelvin Cross Sections

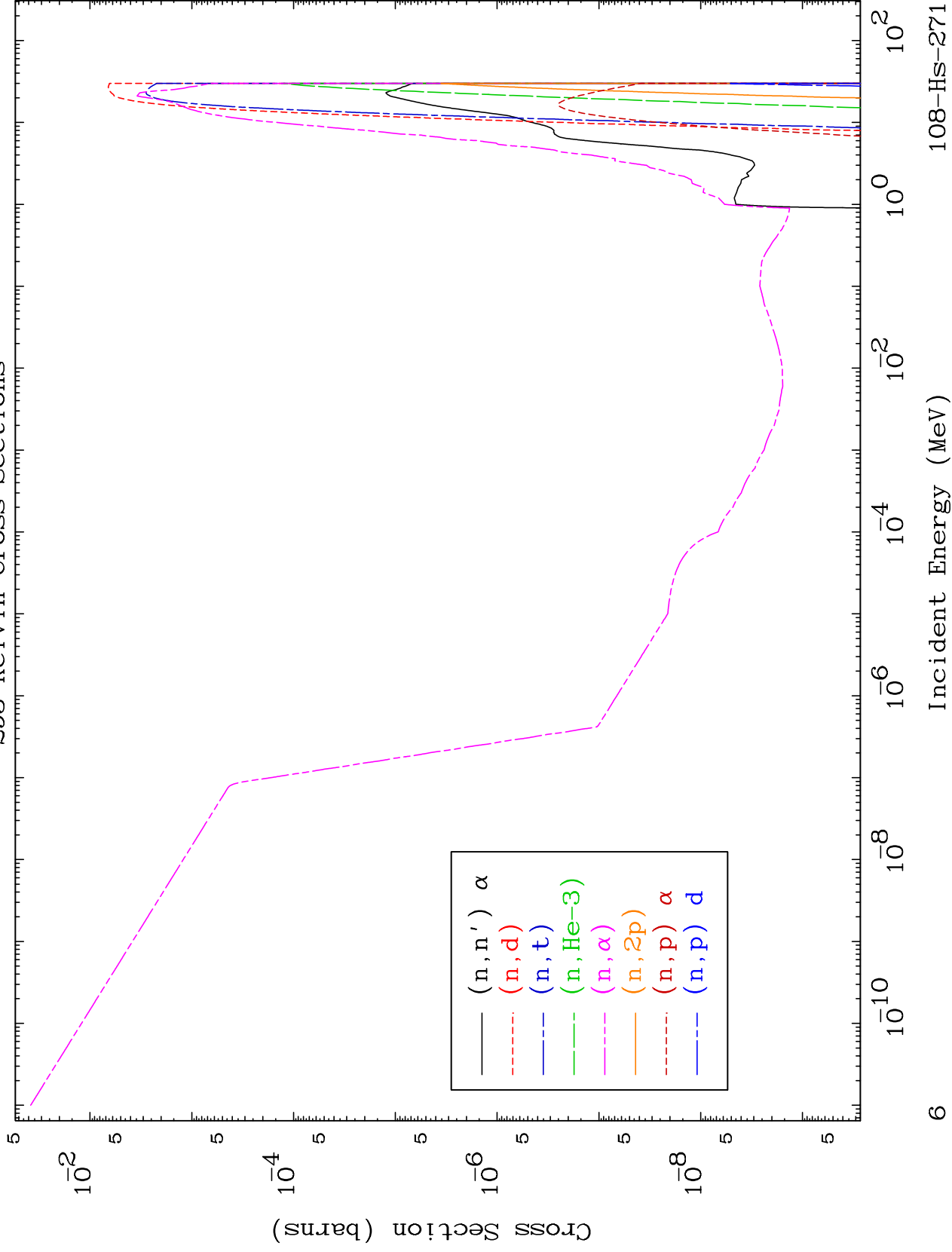
108-Hs-271



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Charged Particle  
293 Kelvin Cross Sections

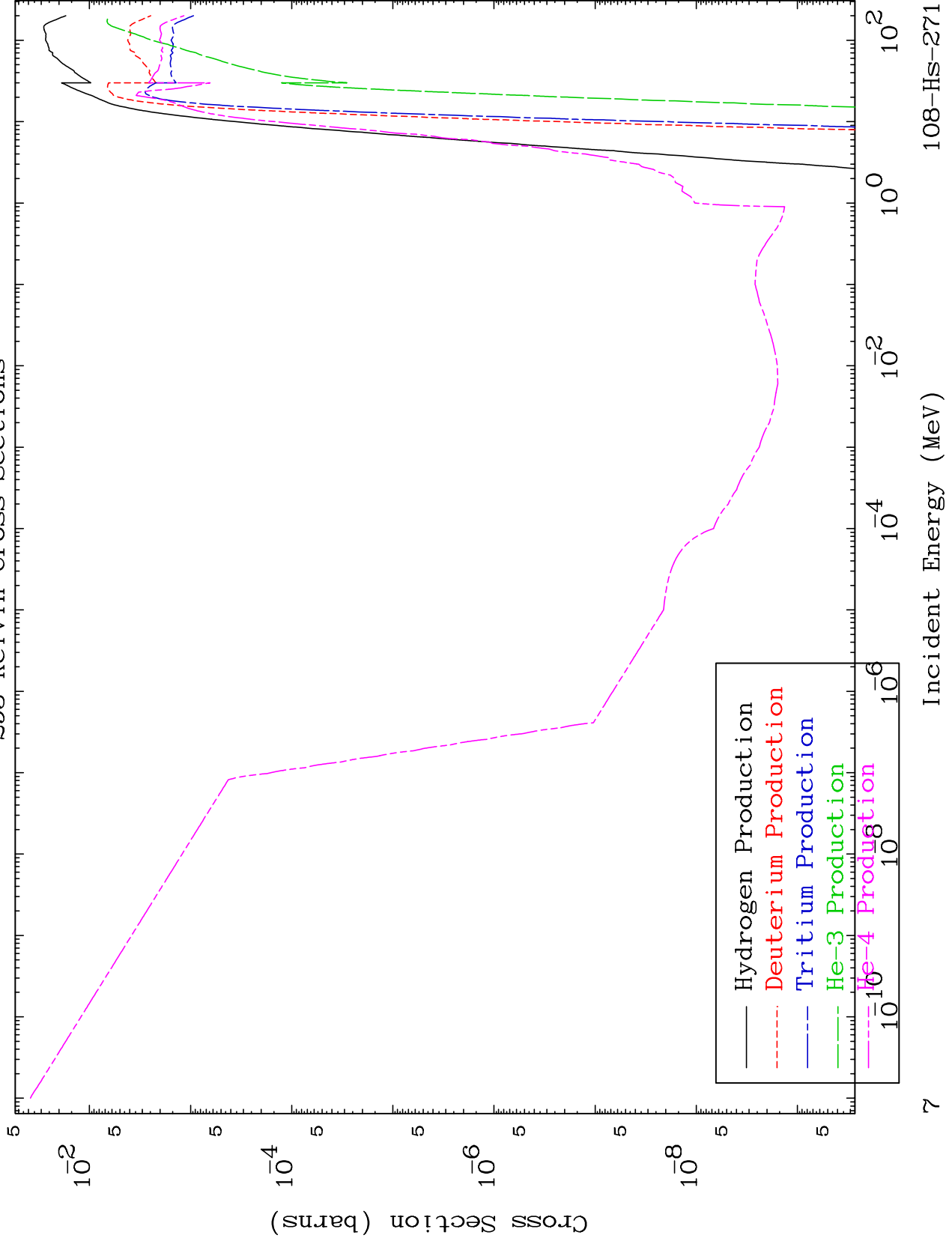
108-Hs-271



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Particle Production  
293 Kelvin Cross Sections

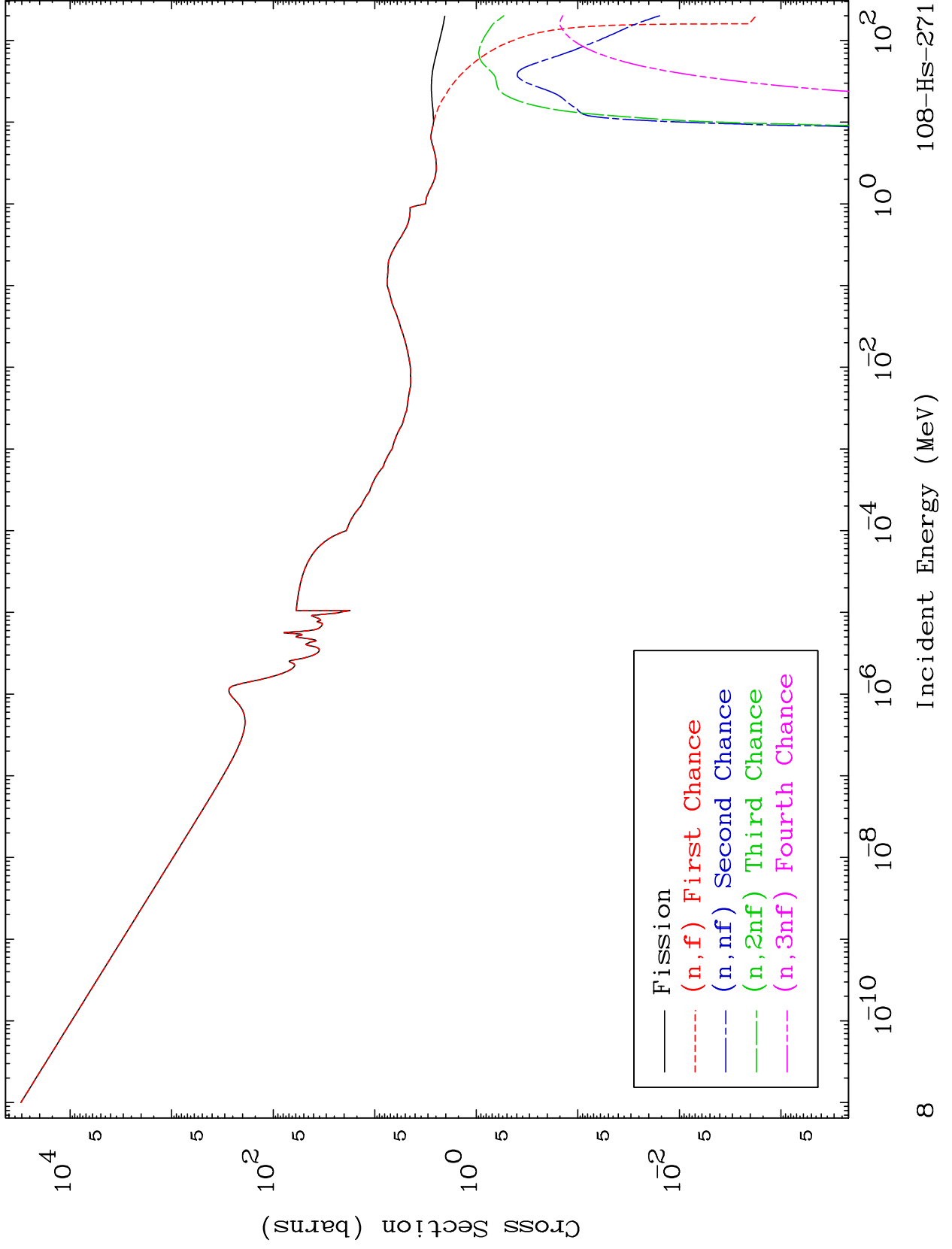
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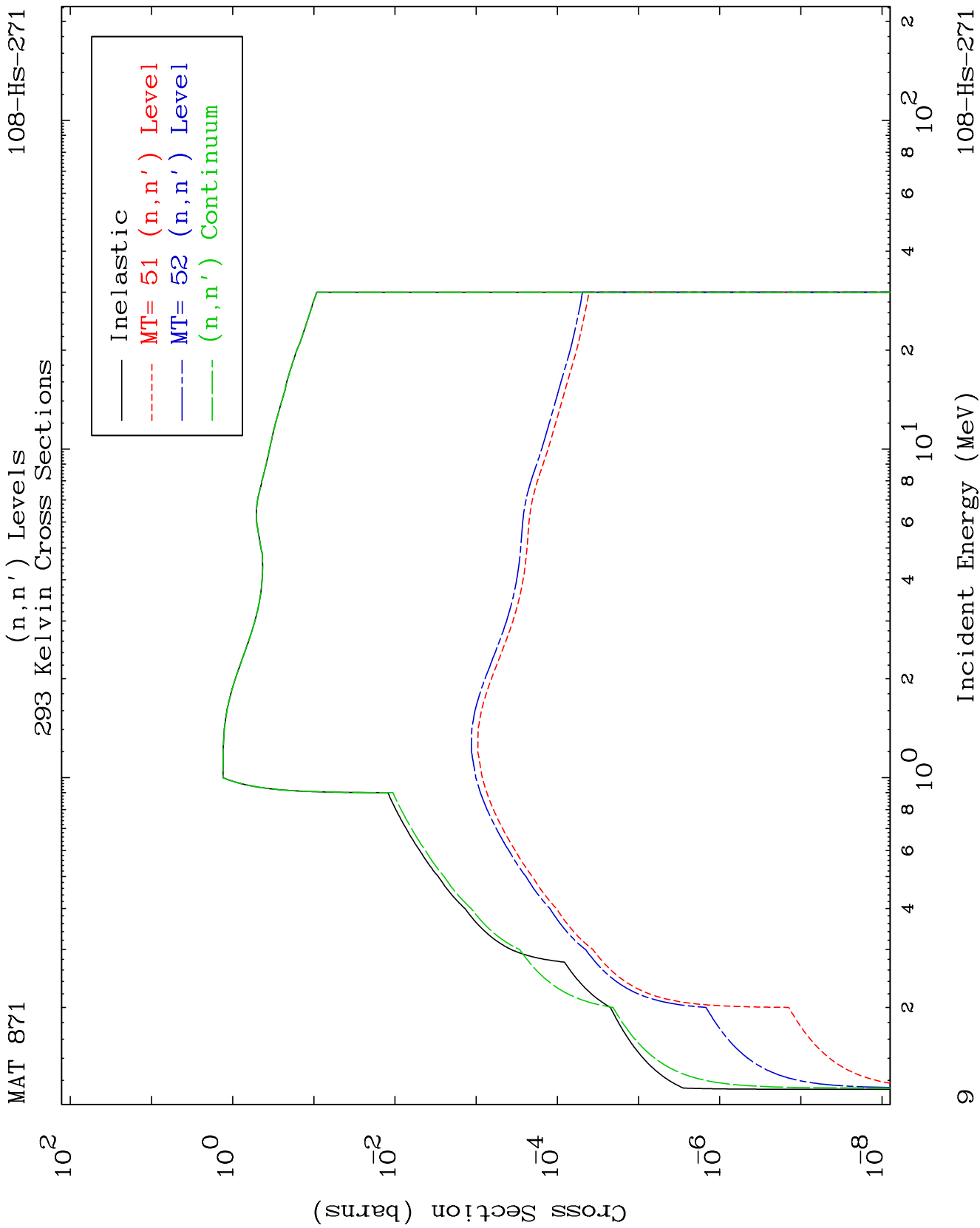


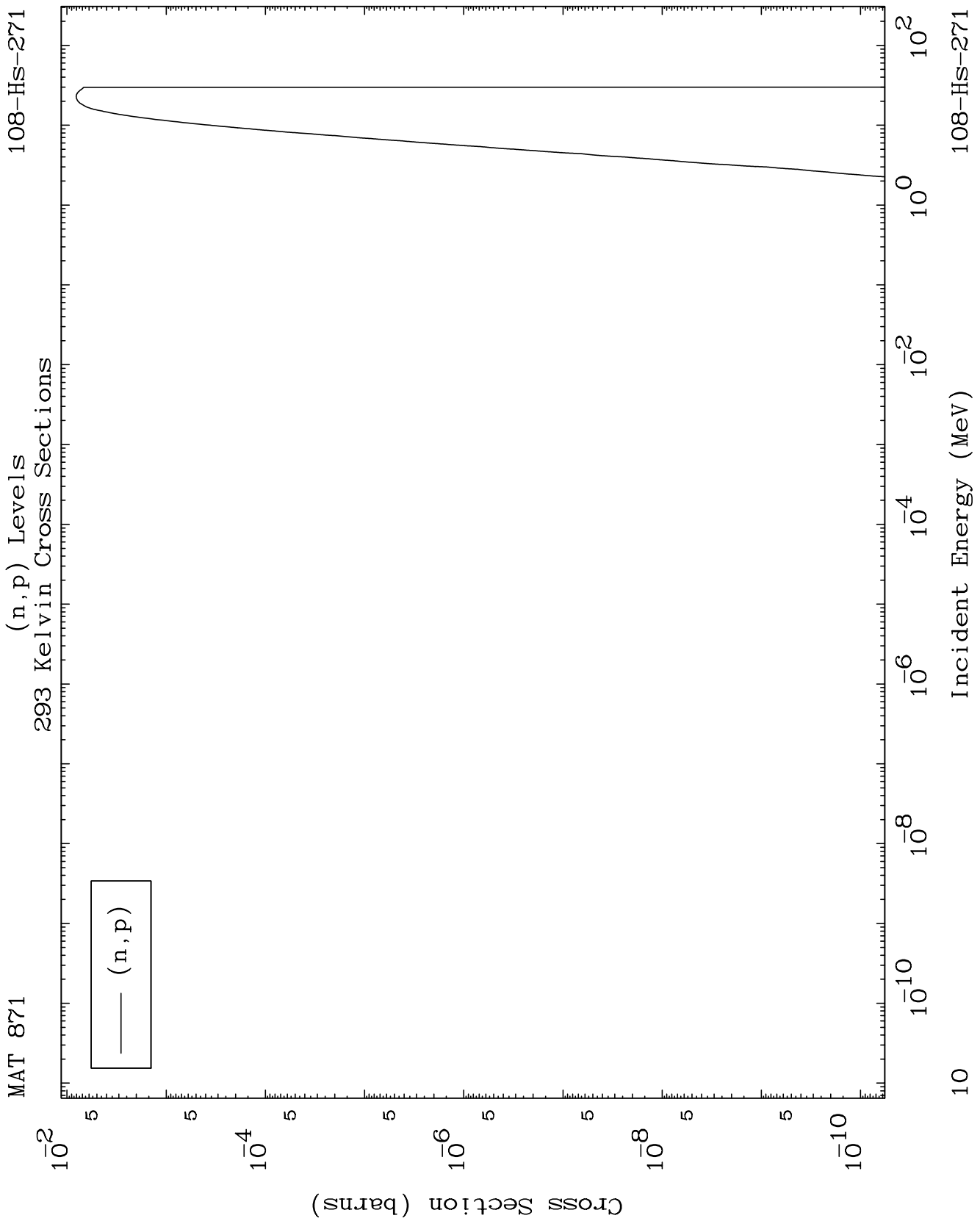
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Fission  
293 Kelvin Cross Sections

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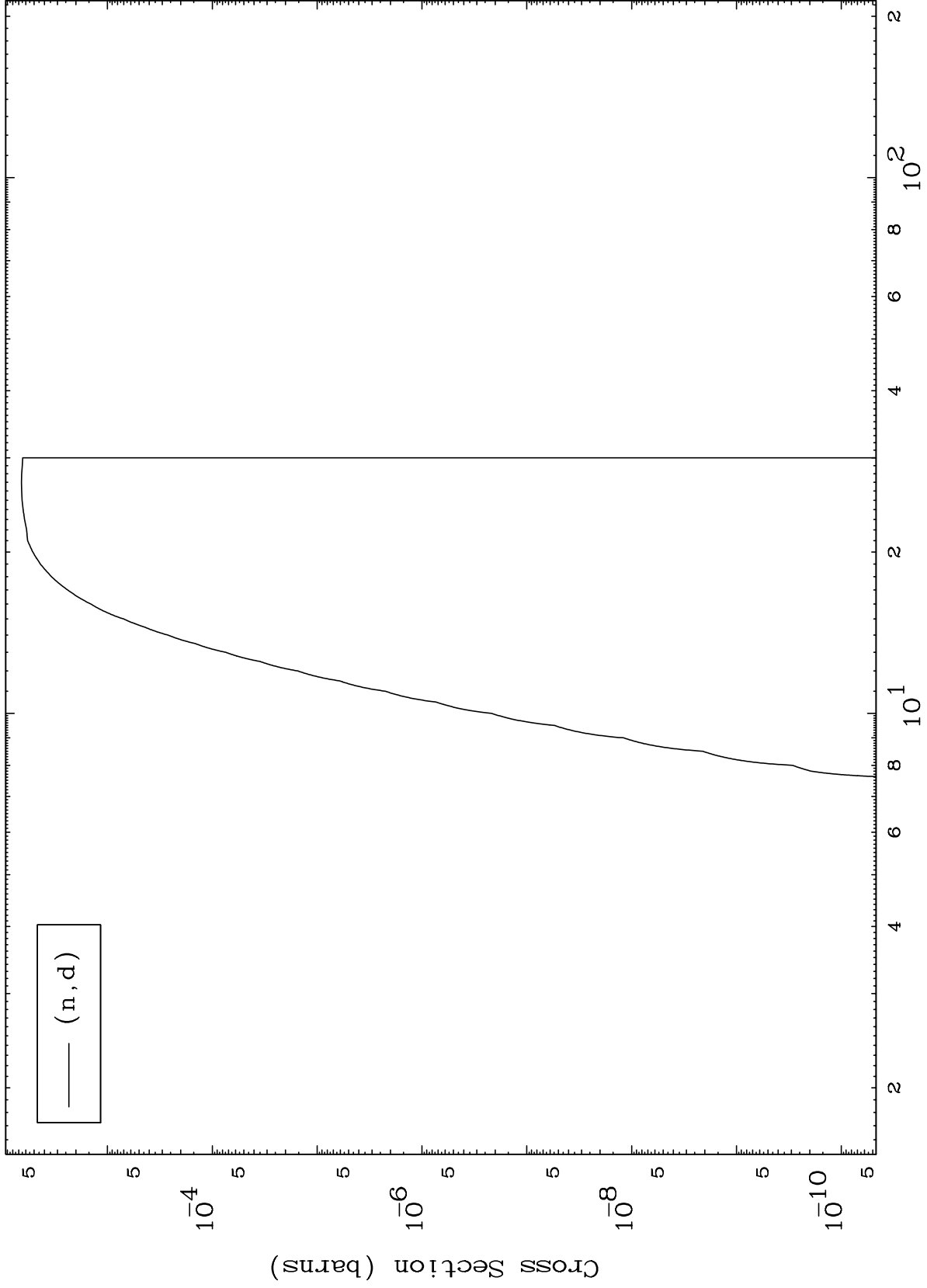




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(n,d) Levels  
293 Kelvin Cross Sections

108-Hs-271



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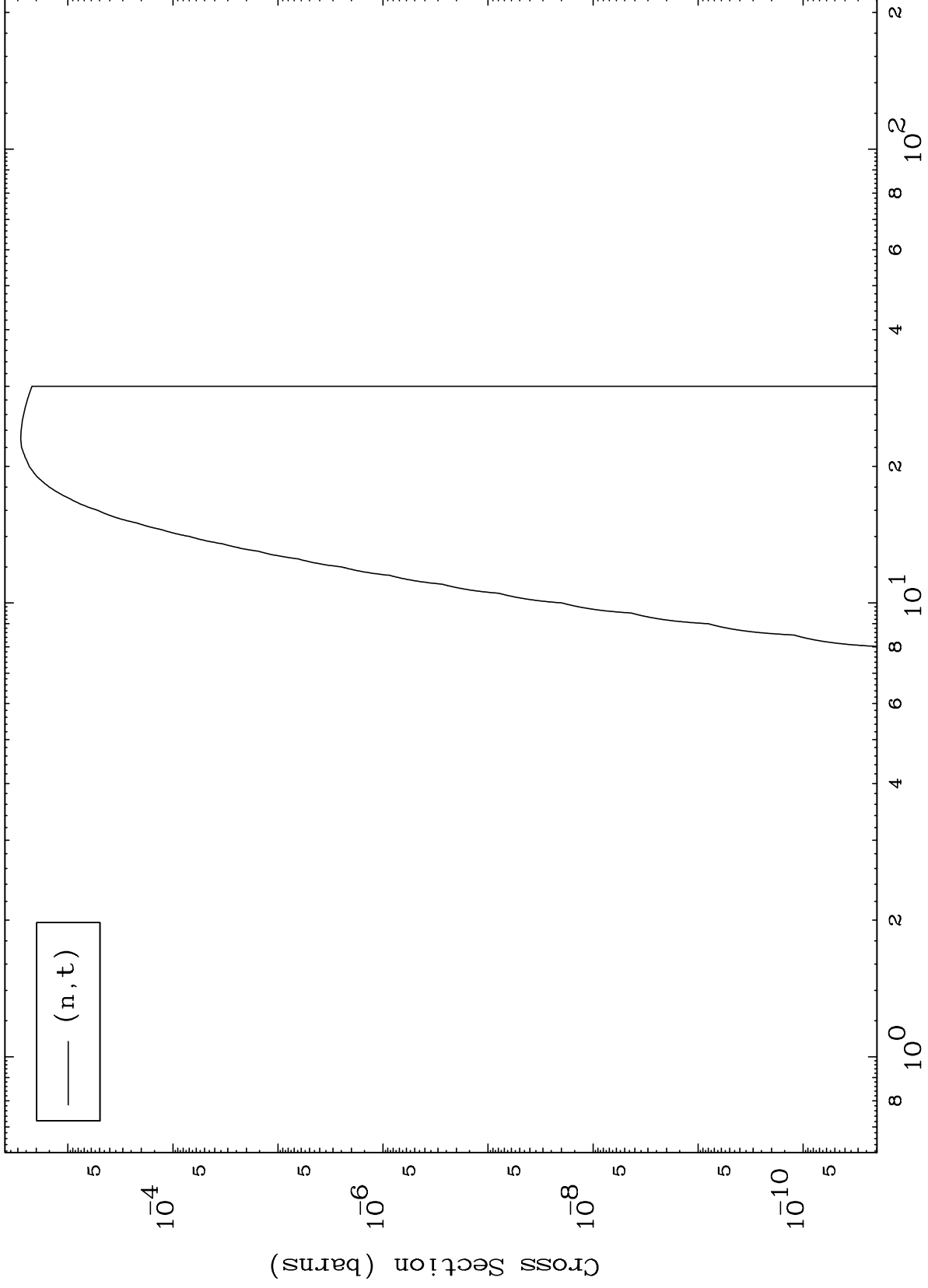
Incident Energy (MeV)

108-Hs-271

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(n,t) Levels  
293 Kelvin Cross Sections

108-Hs-271



12

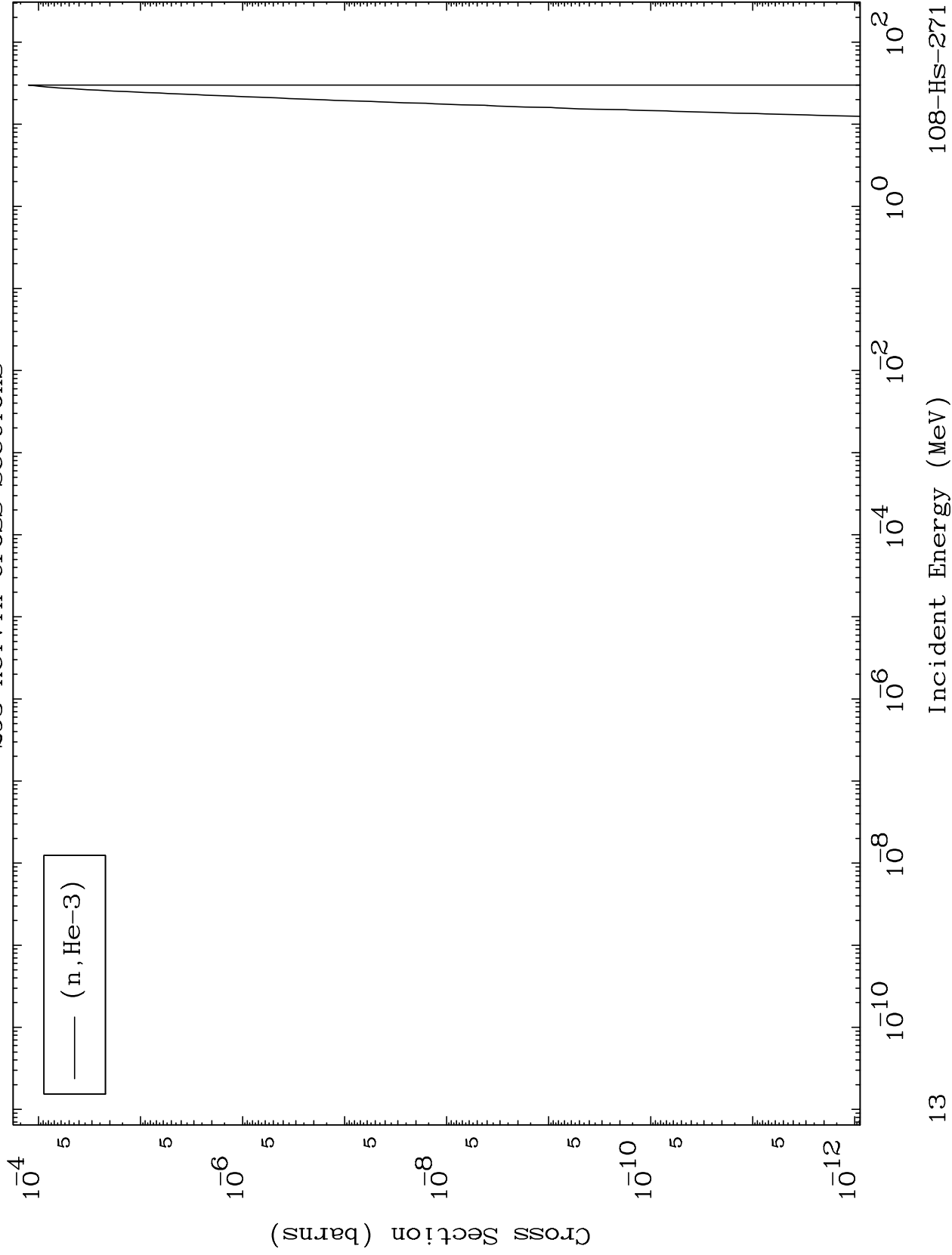
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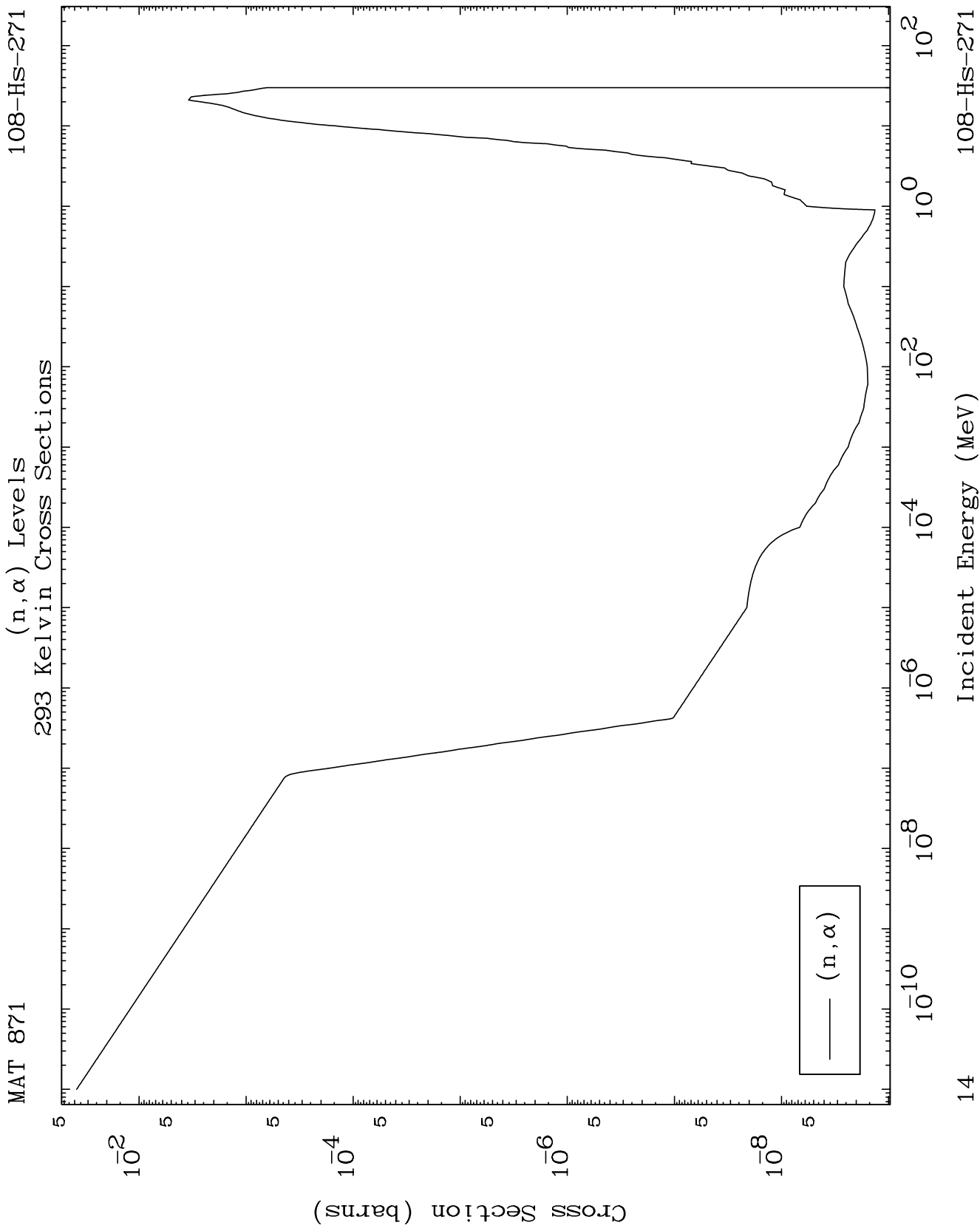
108-Hs-271

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(n,He3) Levels  
293 Kelvin Cross Sections

108-Hs-271

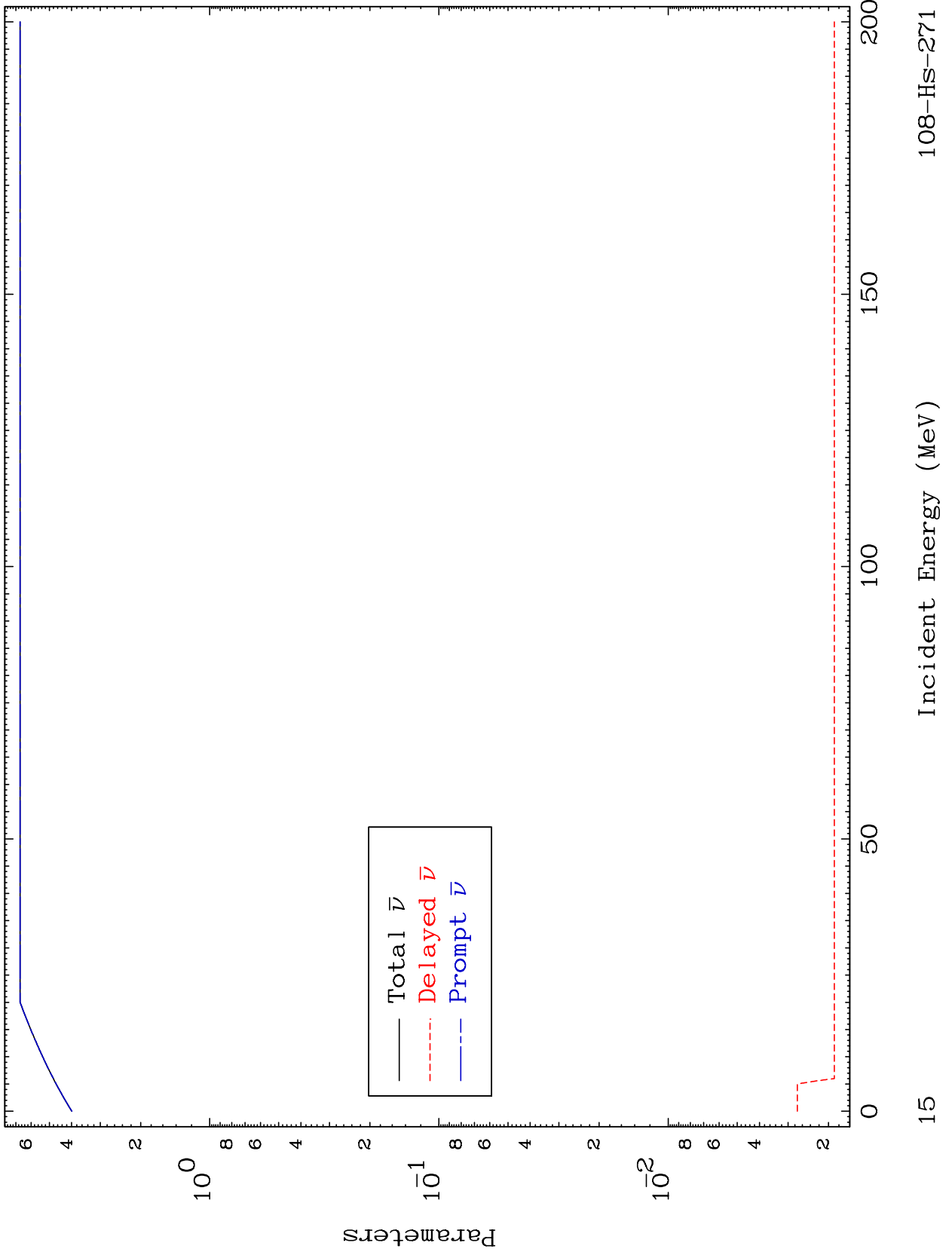




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Energy Release  
Parameters

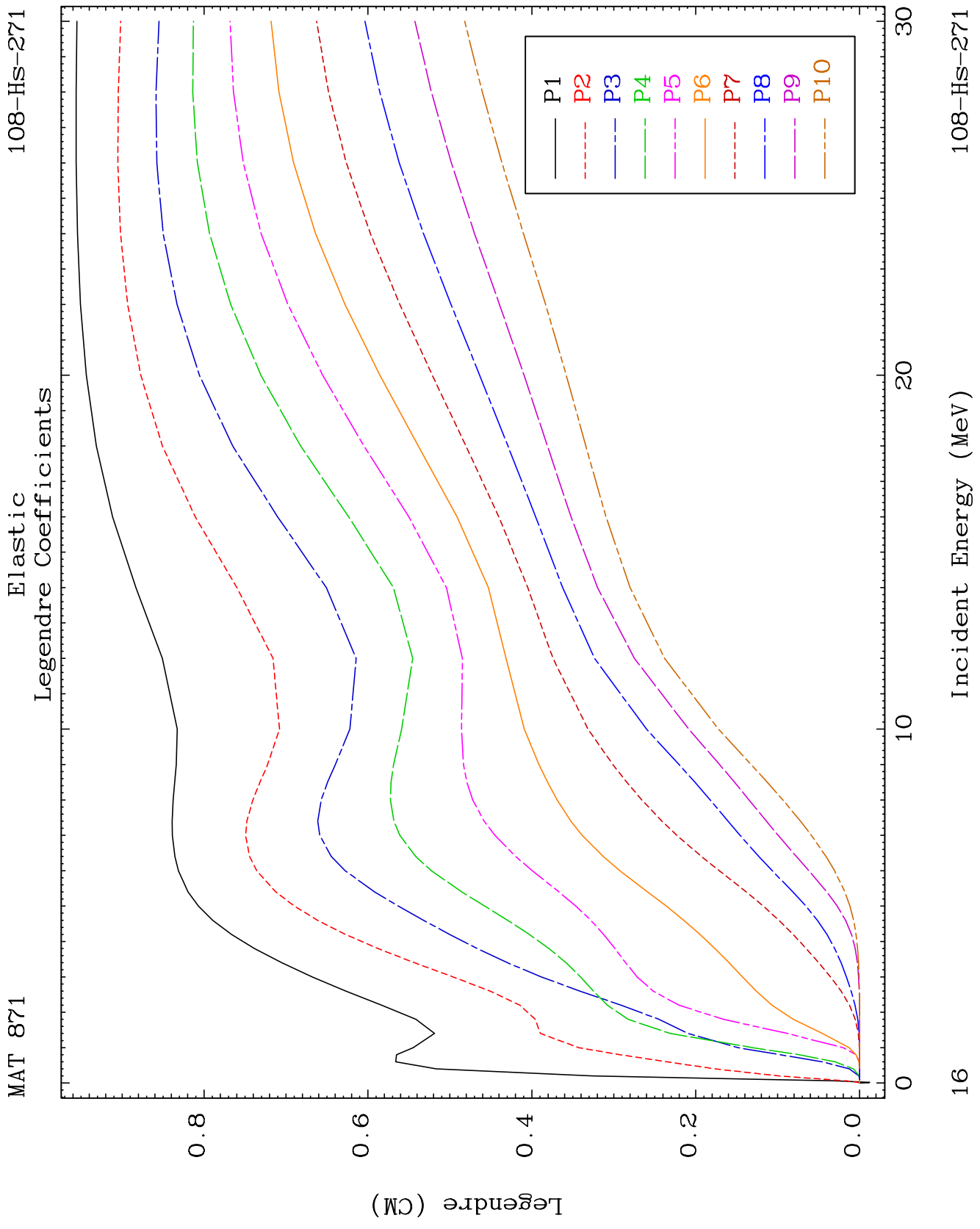
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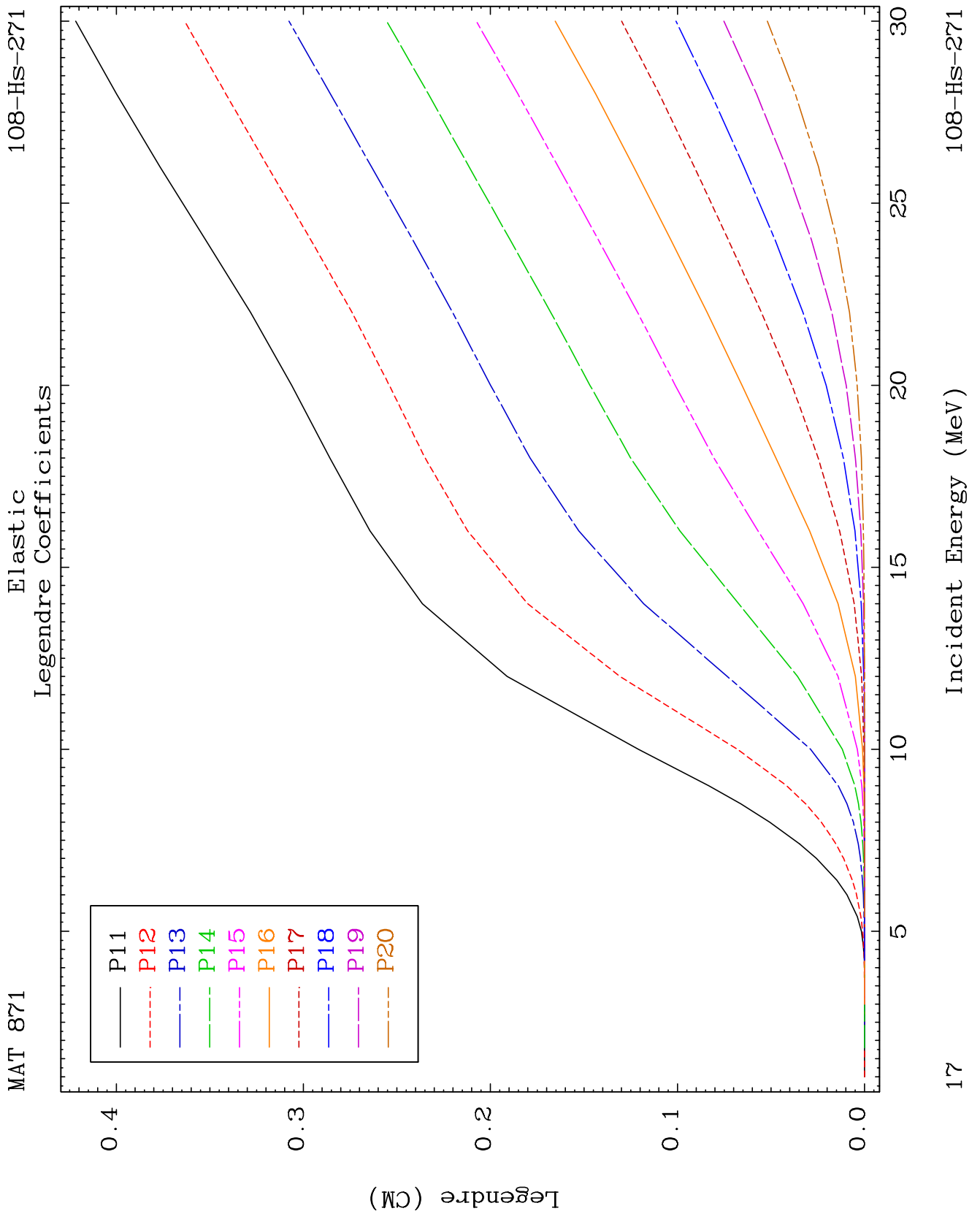


15

Incident Energy (MeV)

108-Hs-271

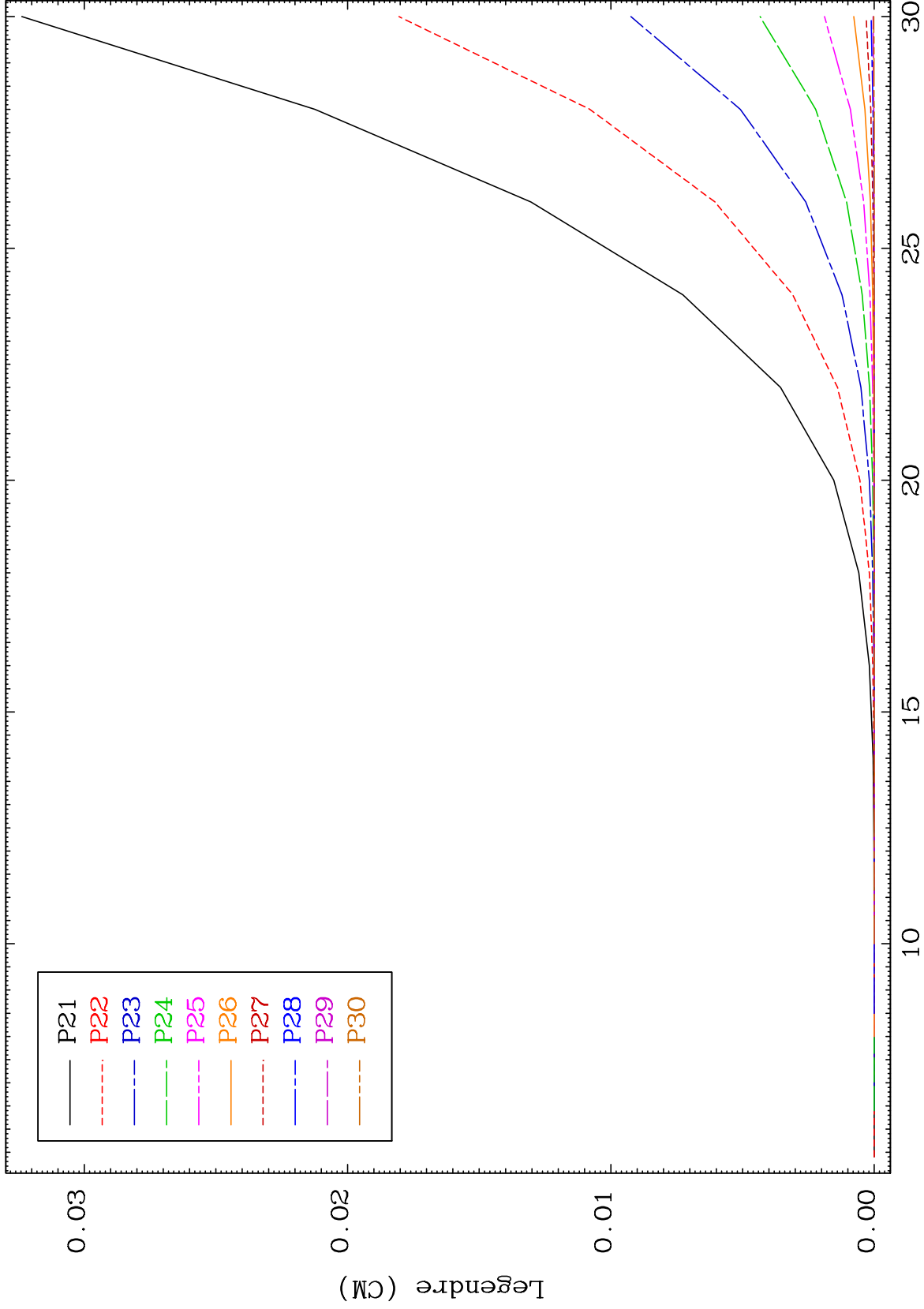




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Elastic  
Legendre Coefficients

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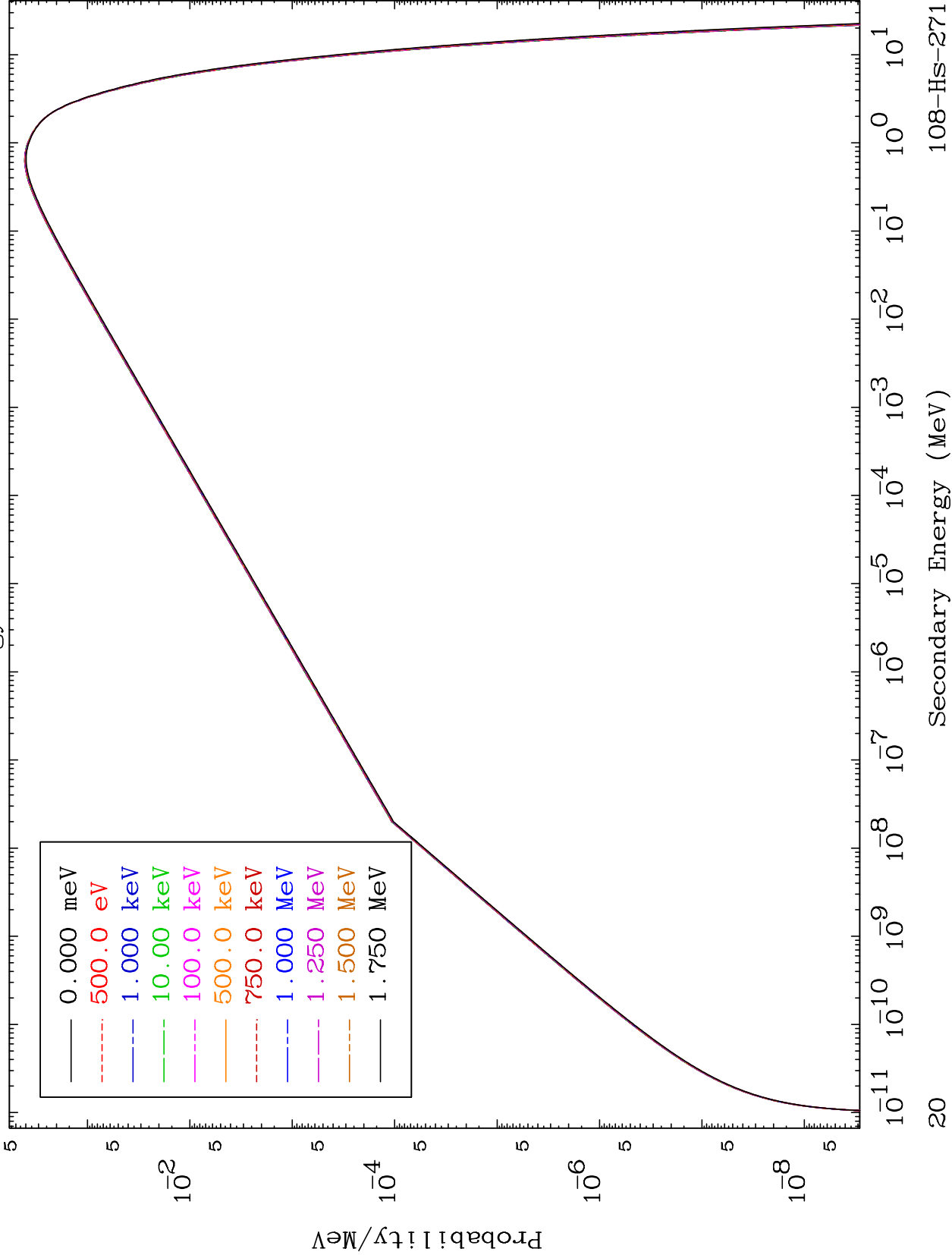
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Incident Energy (MeV)

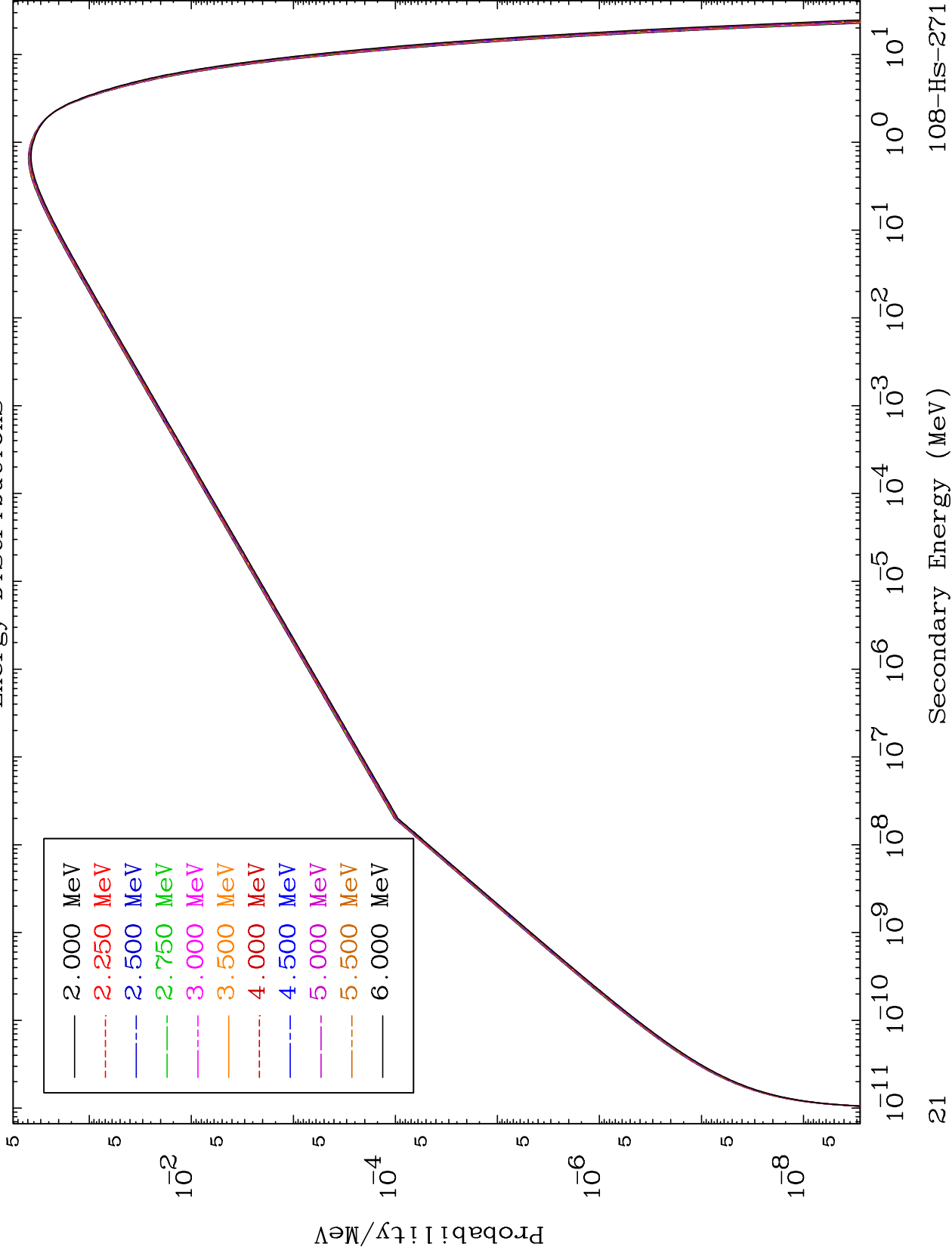
108-Hs-271



Fission Energy Distributions



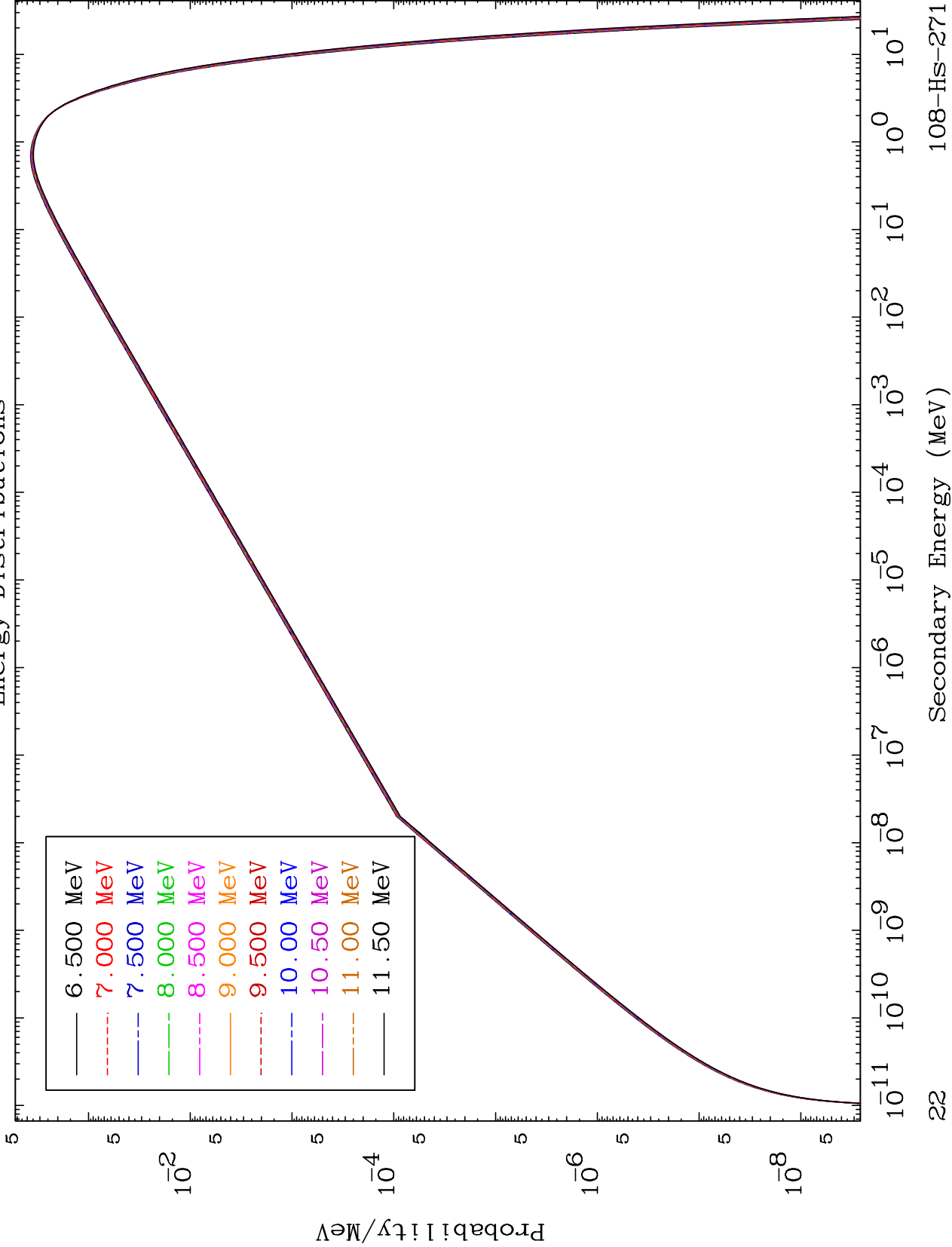
Fission Energy Distributions



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Fission Energy Distributions

108-Hs-271



108-Hs-271

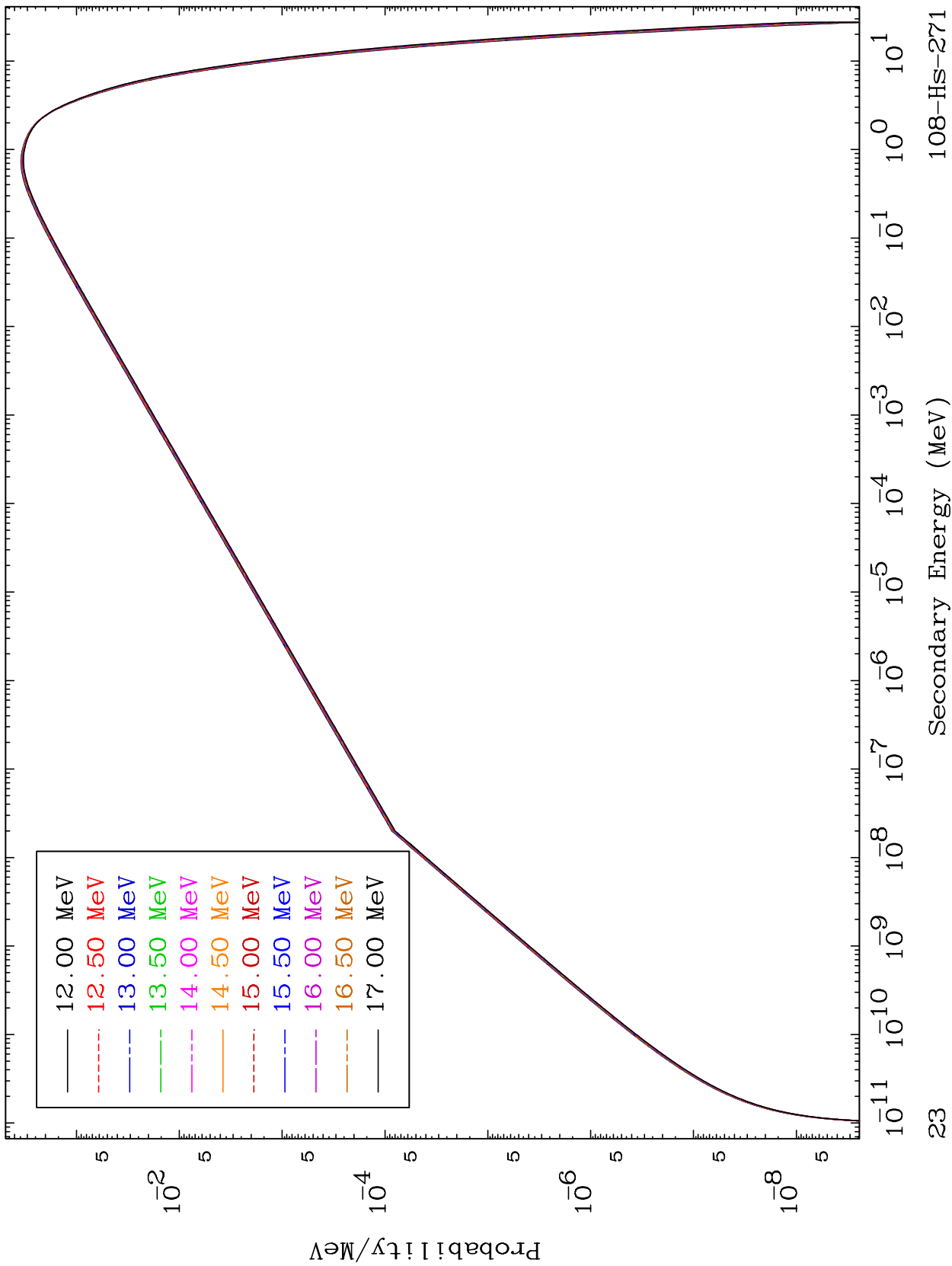
Secondary Energy (MeV)

22

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### Fission Energy Distributions

108-Hs-271



108-Hs-271

Secondary Energy (MeV)

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### Fission Energy Distributions

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