

Program EVALPLOT
(Version 2021-1)

by

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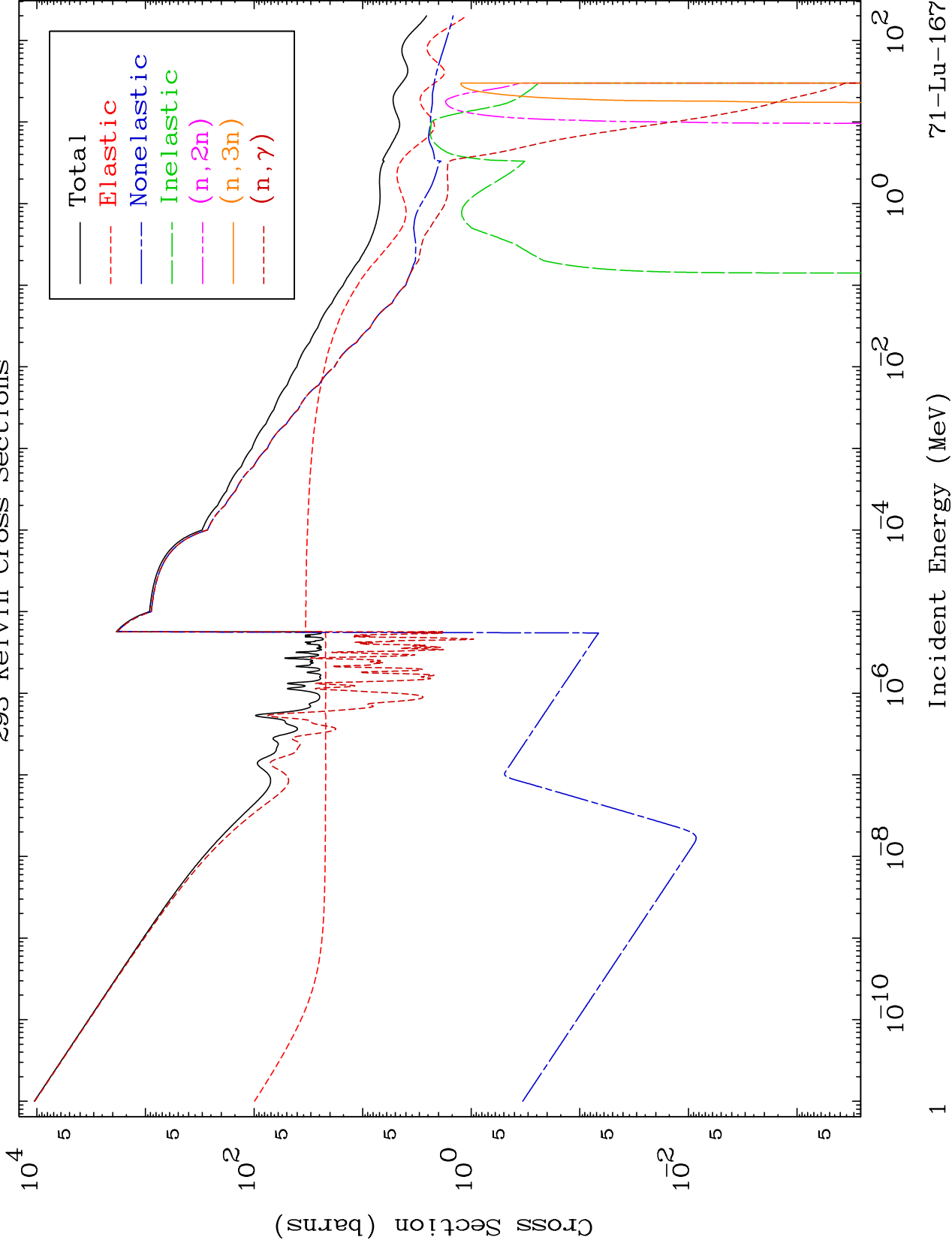
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Press Mouse Button to Start

MAT 7101

Neutron Major
293 Kelvin Cross Sections

71-Lu-167



1

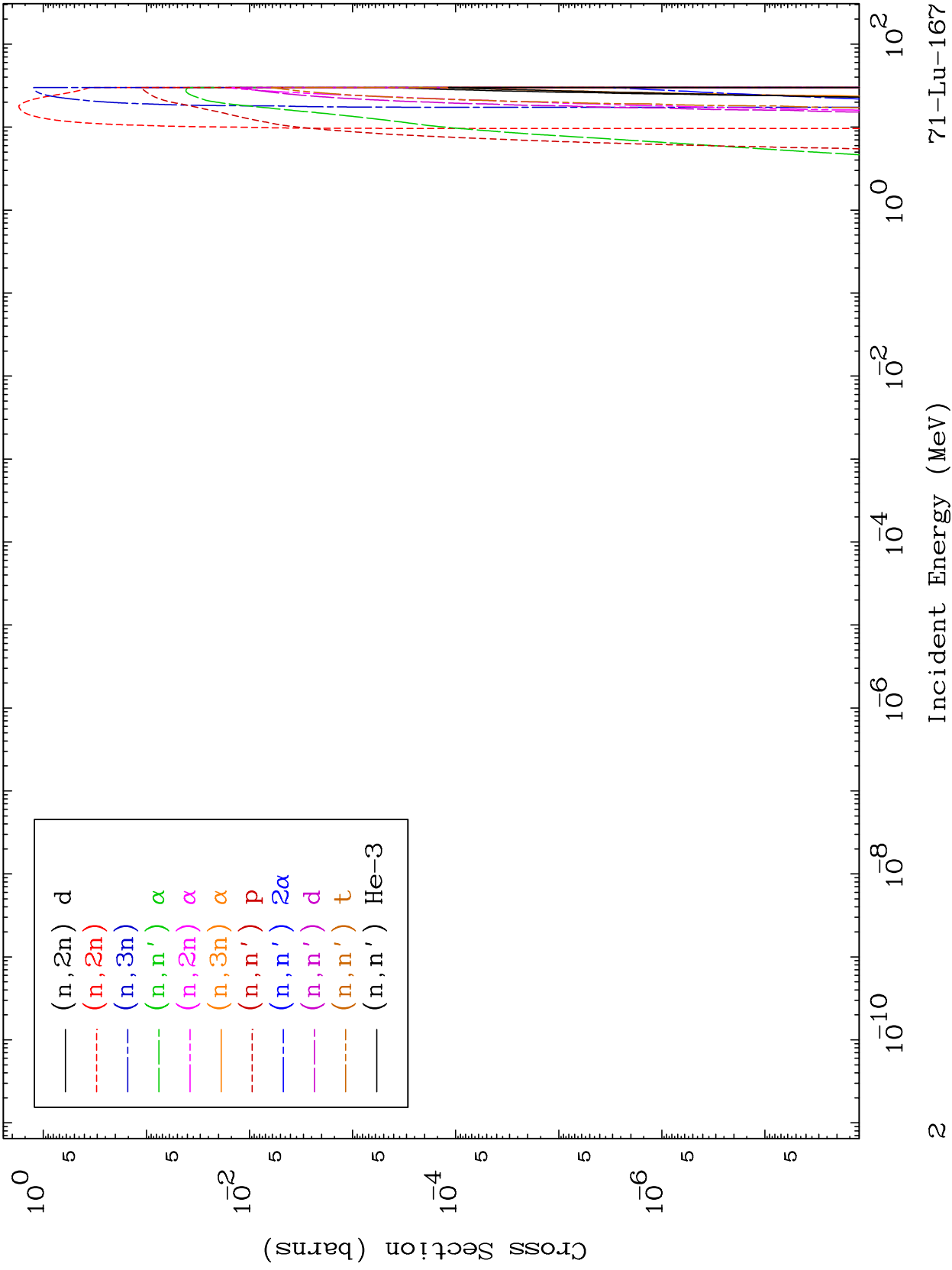
Incident Energy (MeV)

71-Lu-167

MAT 7101

Neutron Absorption
293 Kelvin Cross Sections

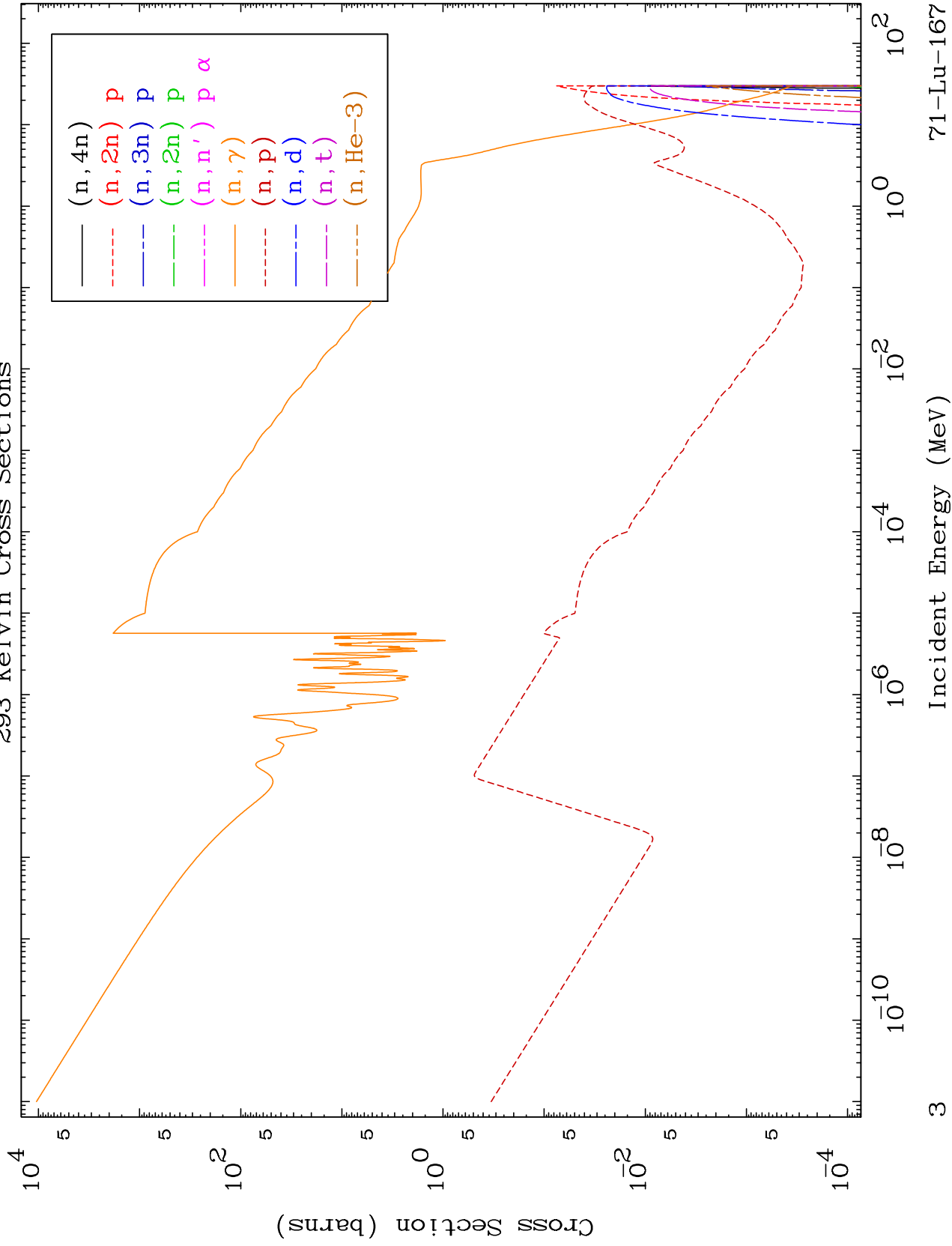
71-Lu-167



MAT 7101

Neutron Absorption
293 Kelvin Cross Sections

71-Lu-167



71-Lu-167

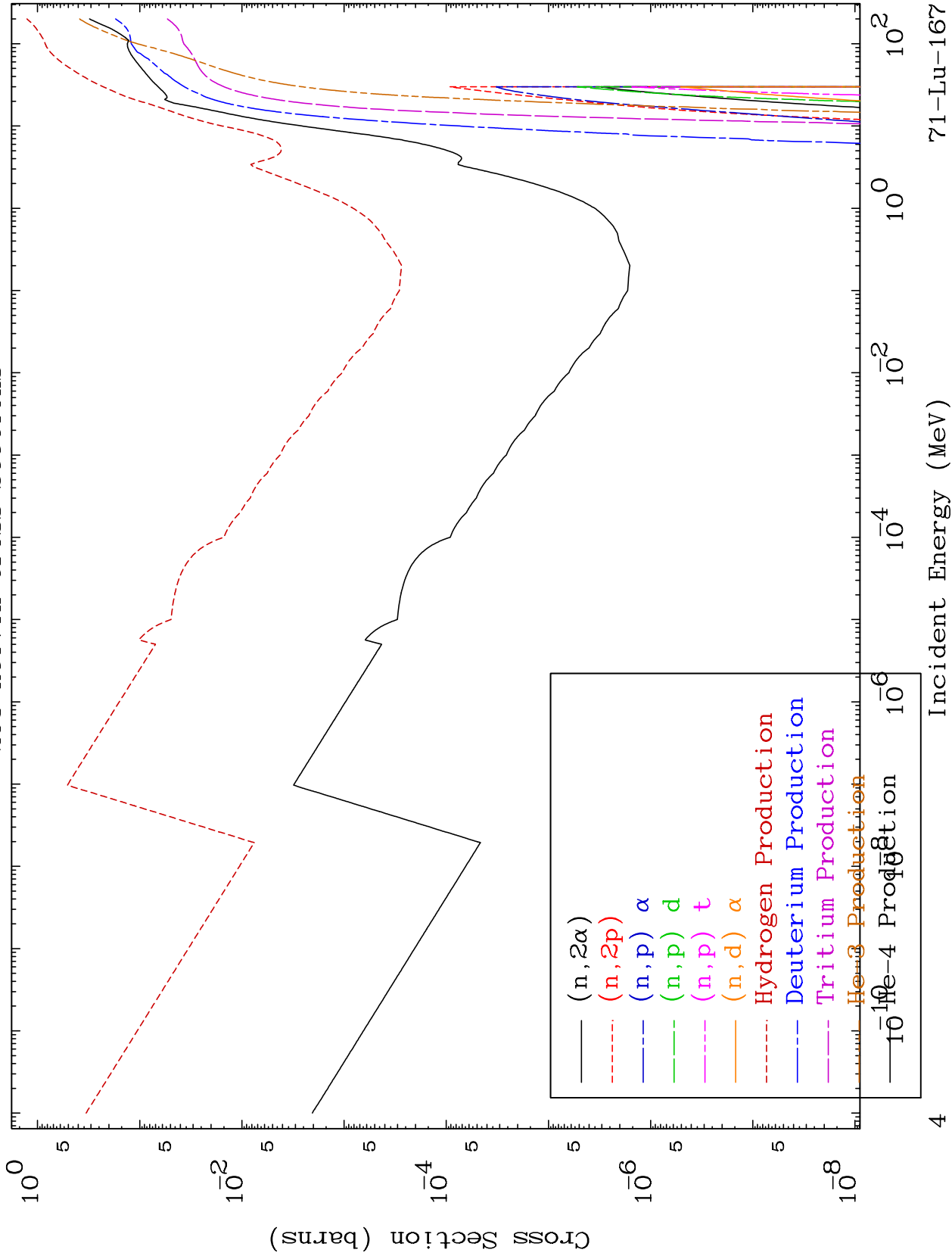
Incident Energy (MeV)

3

MAT 7101

Neutron Absorption
293 Kelvin Cross Sections

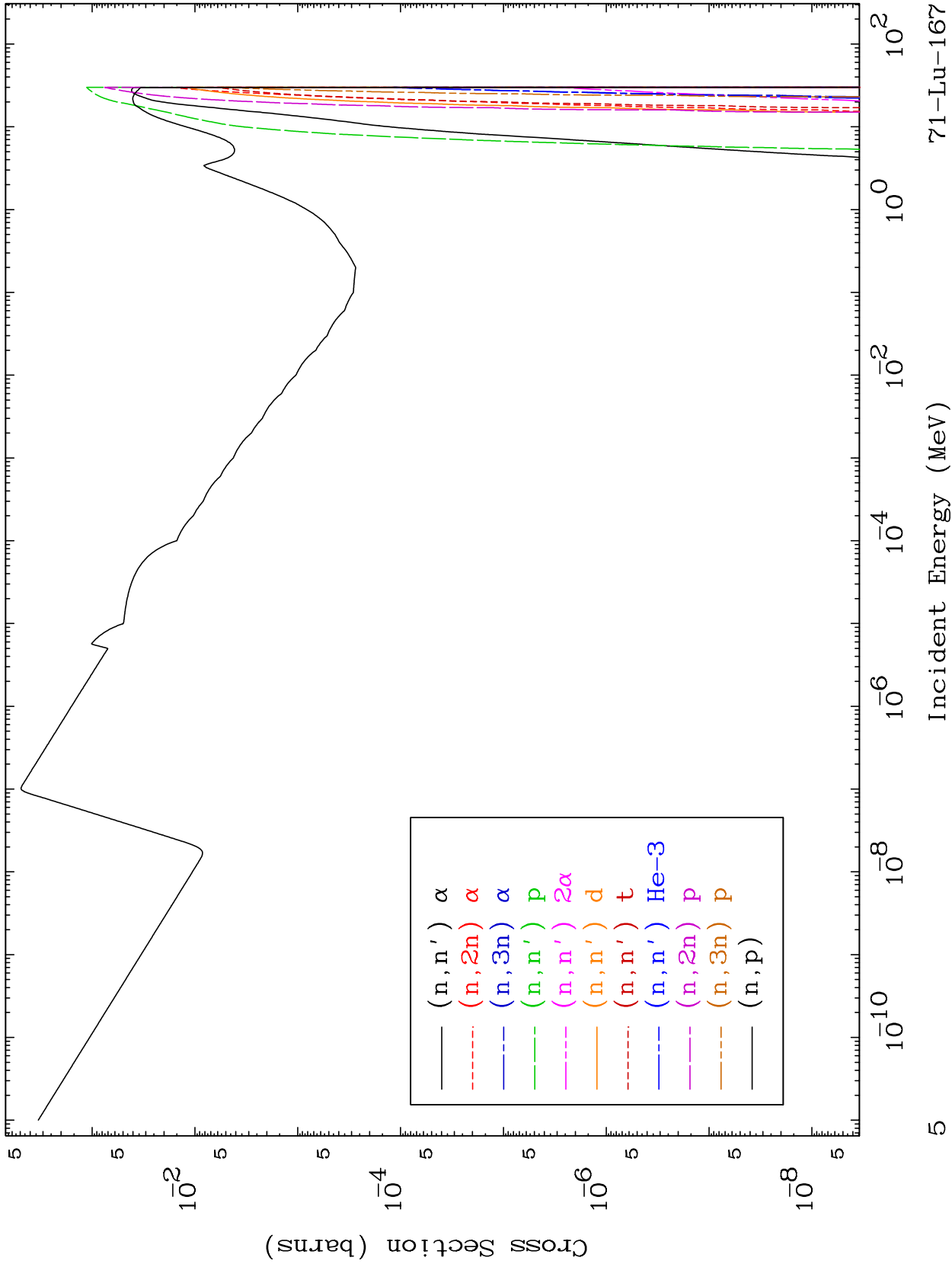
71-Lu-167



MAT 7101

Charged Particle
293 Kelvin Cross Sections

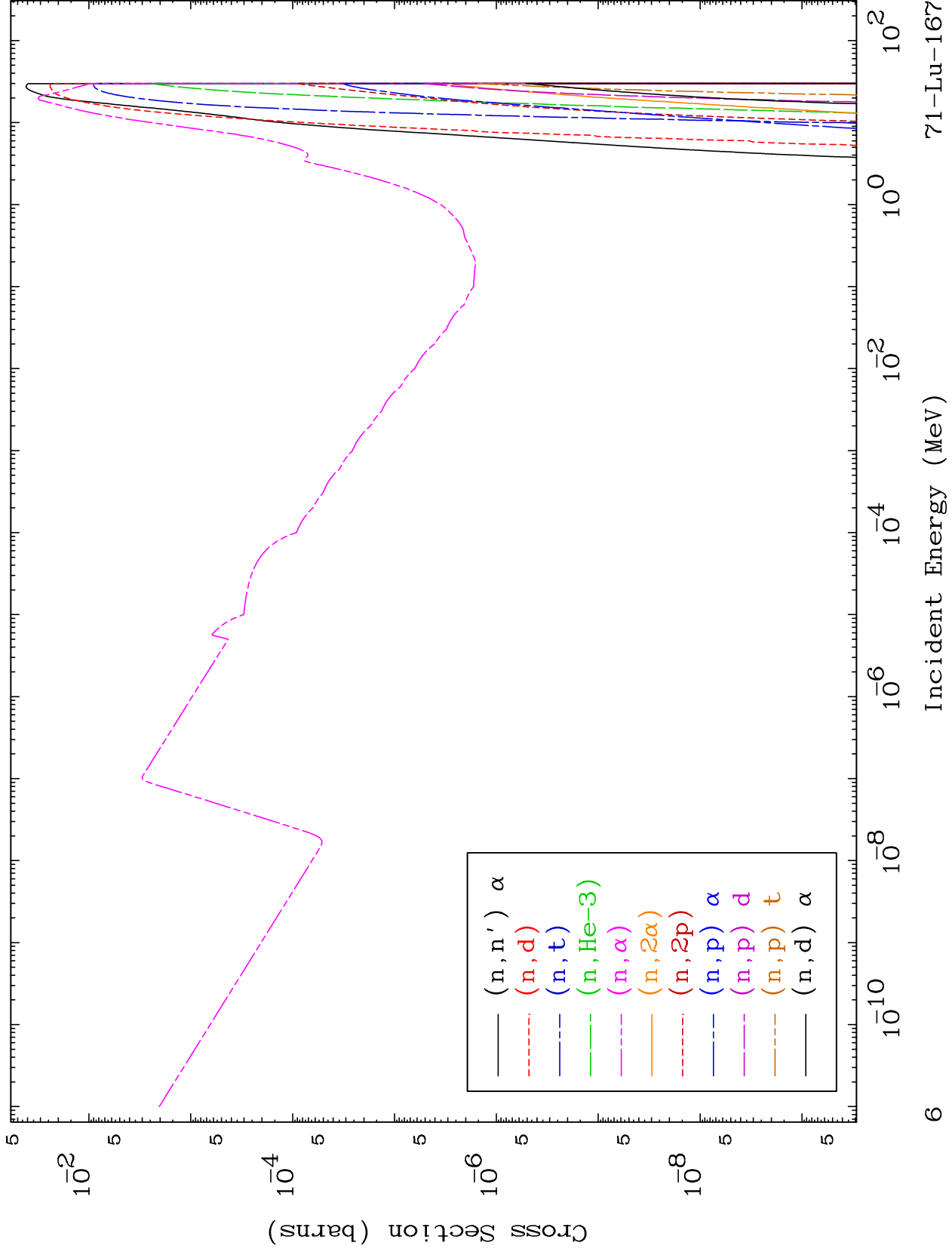
71-Lu-167



MAT 7101

Charged Particle
293 Kelvin Cross Sections

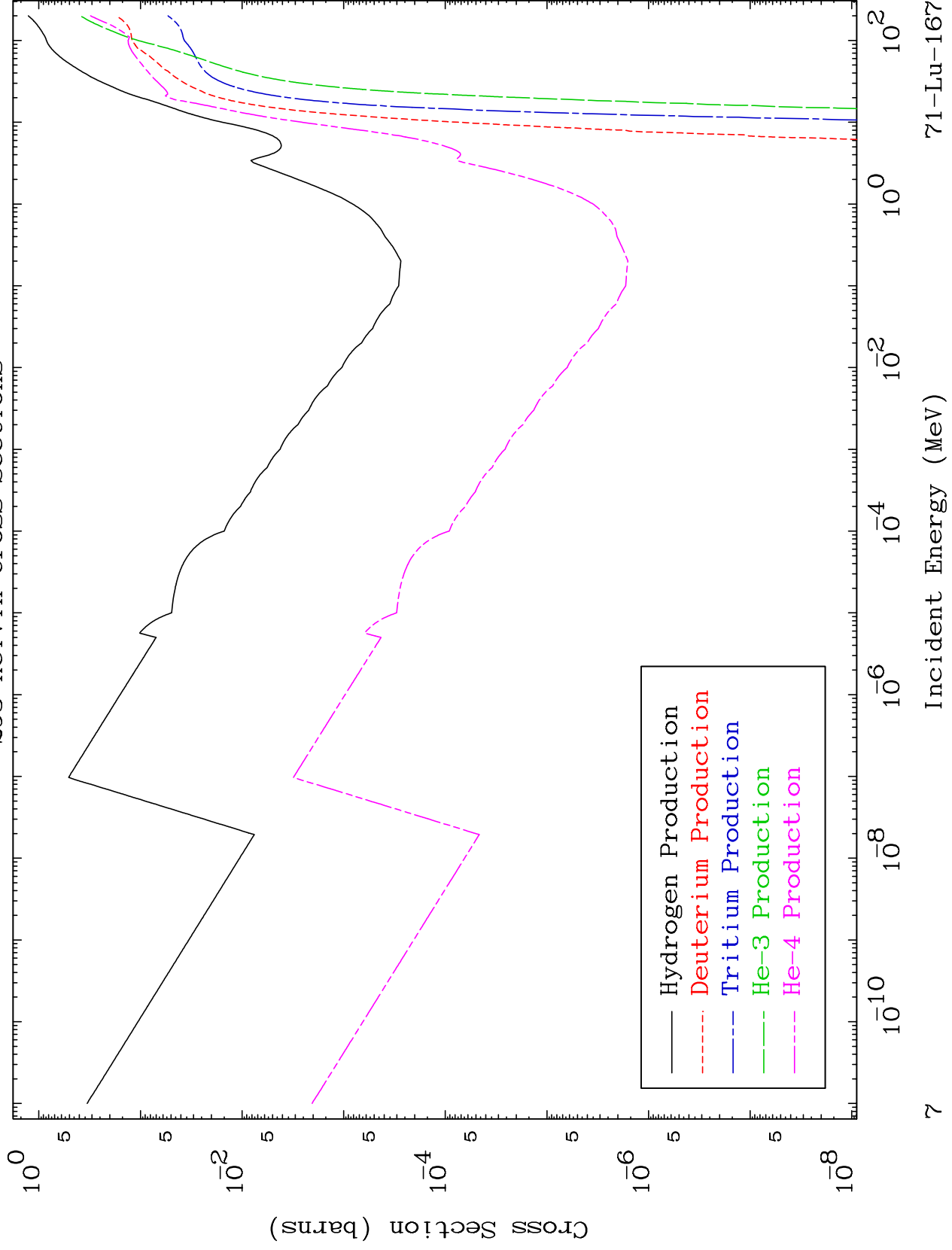
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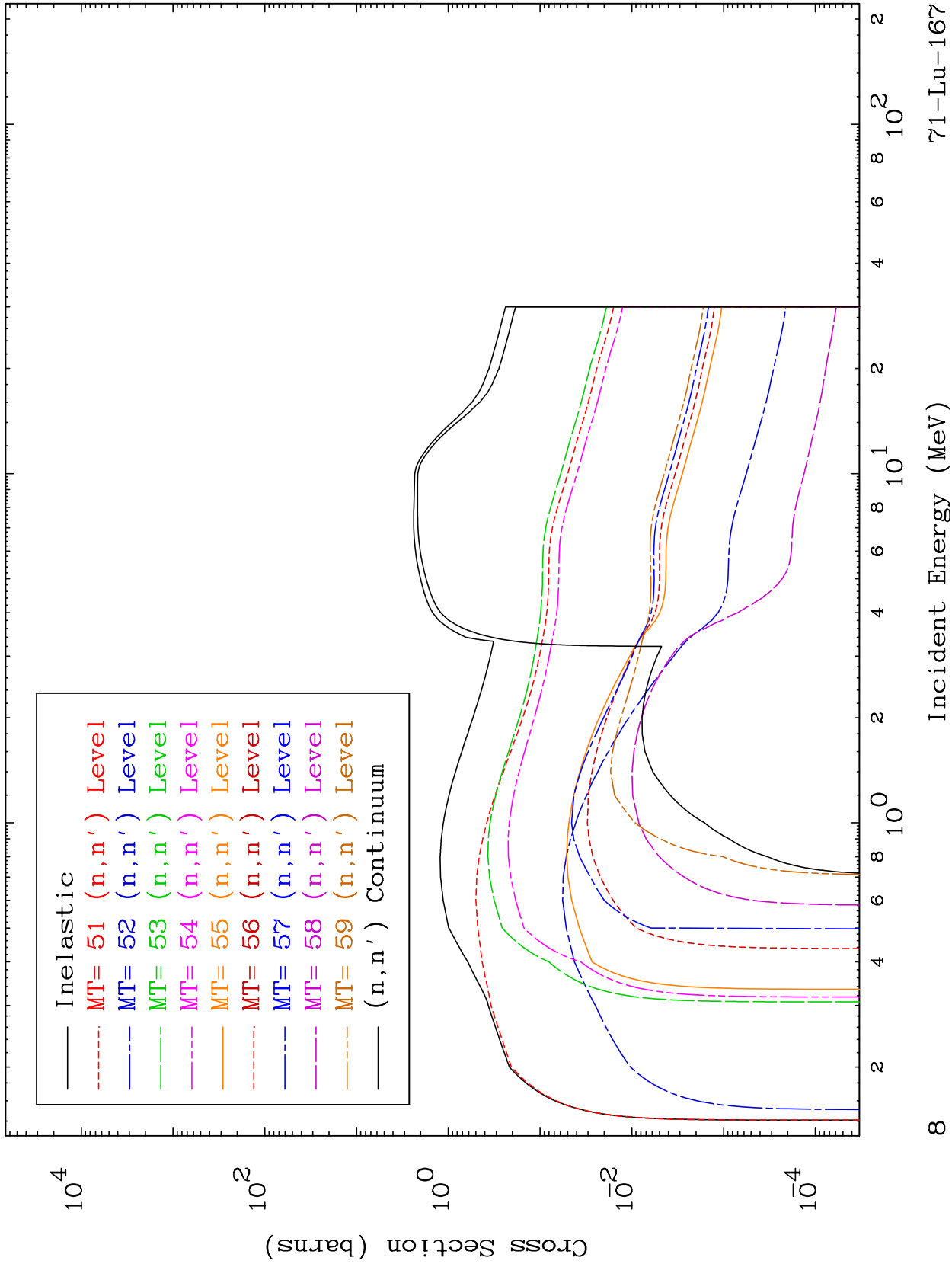


MAT 7101

Particle Production
293 Kelvin Cross Sections

71-Lu-167

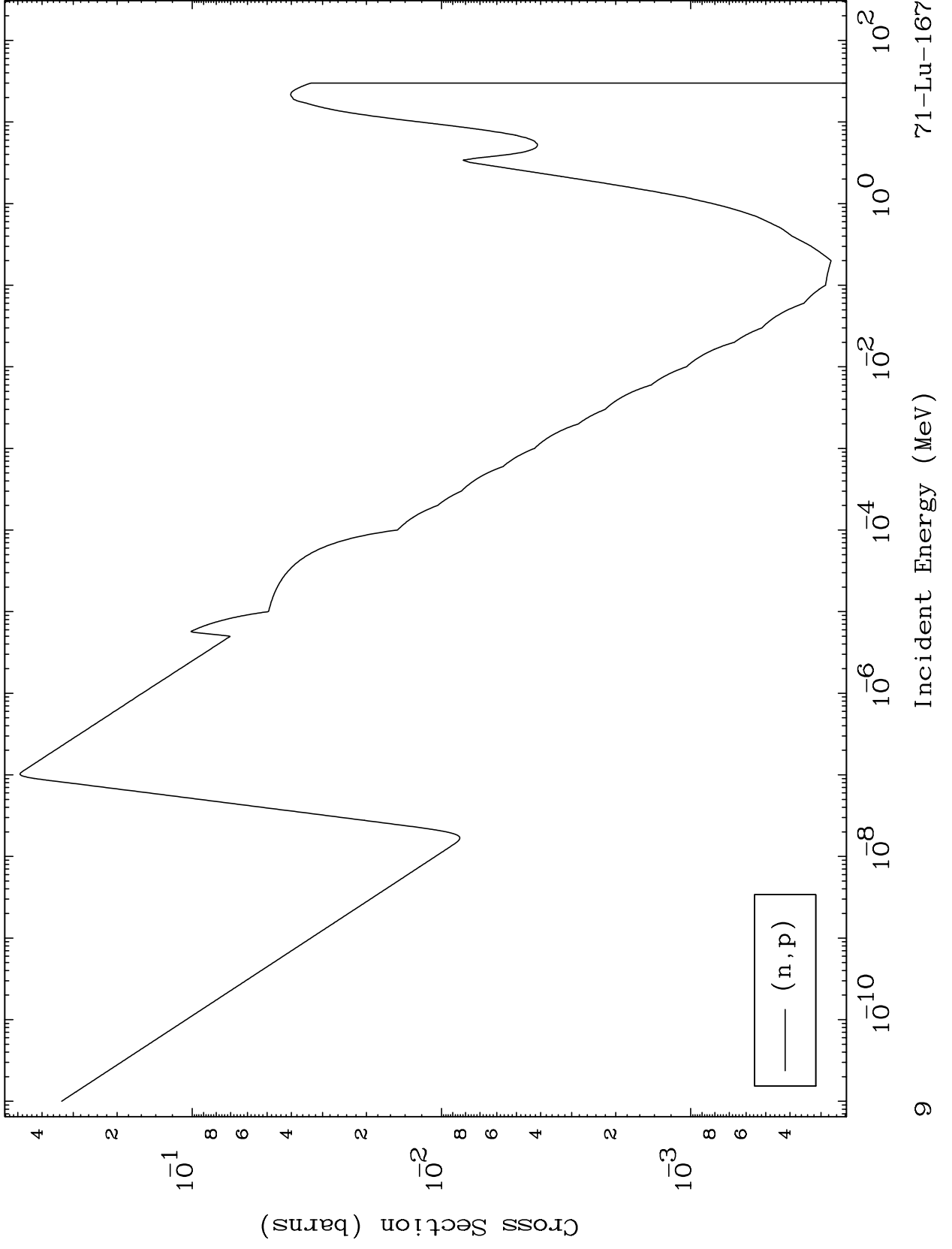




MAT 7101

(n,p) Levels
293 Kelvin Cross Sections

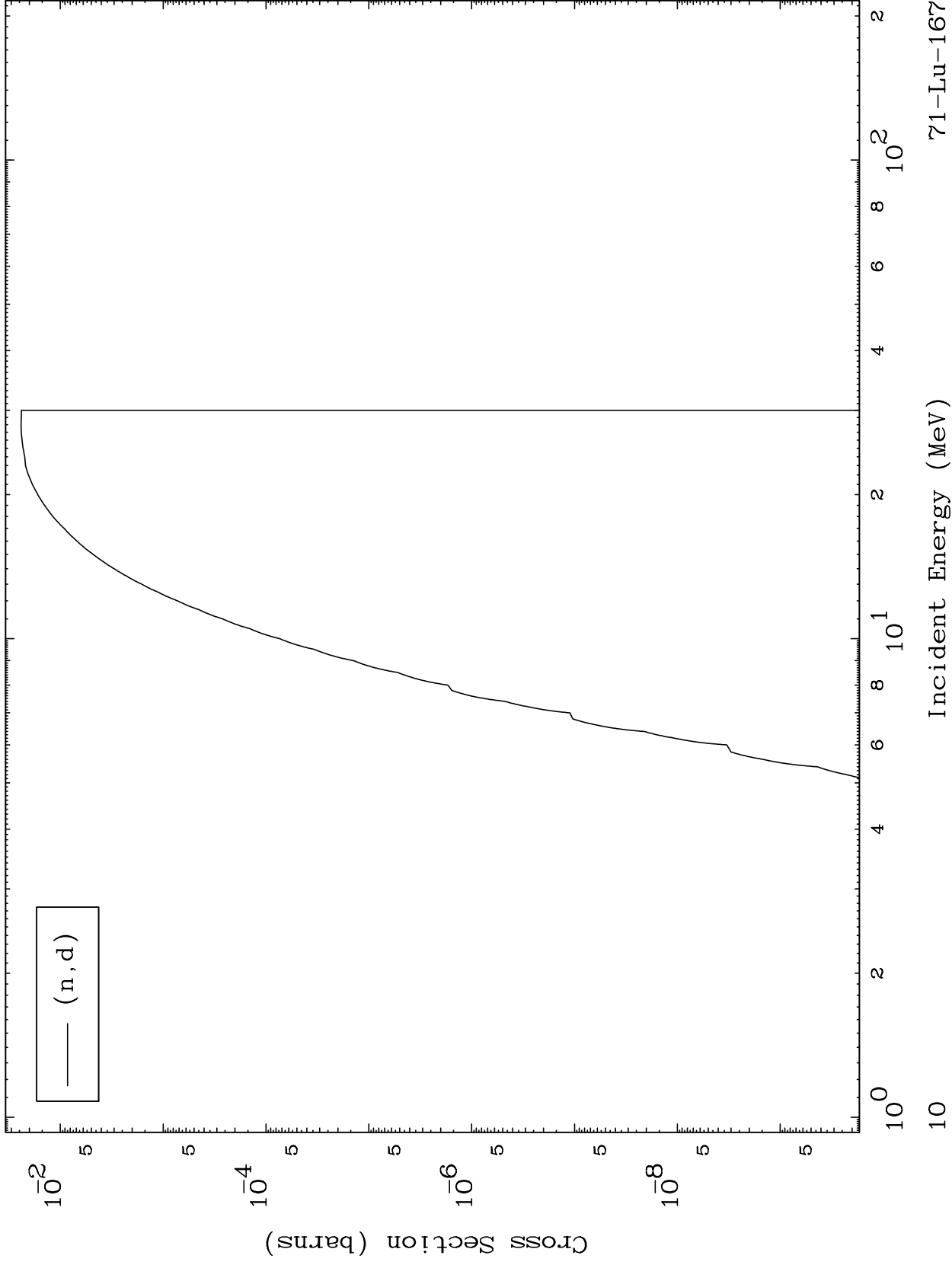
71-Lu-167



MAT 7101

(n,d) Levels
293 Kelvin Cross Sections

71-Lu-167



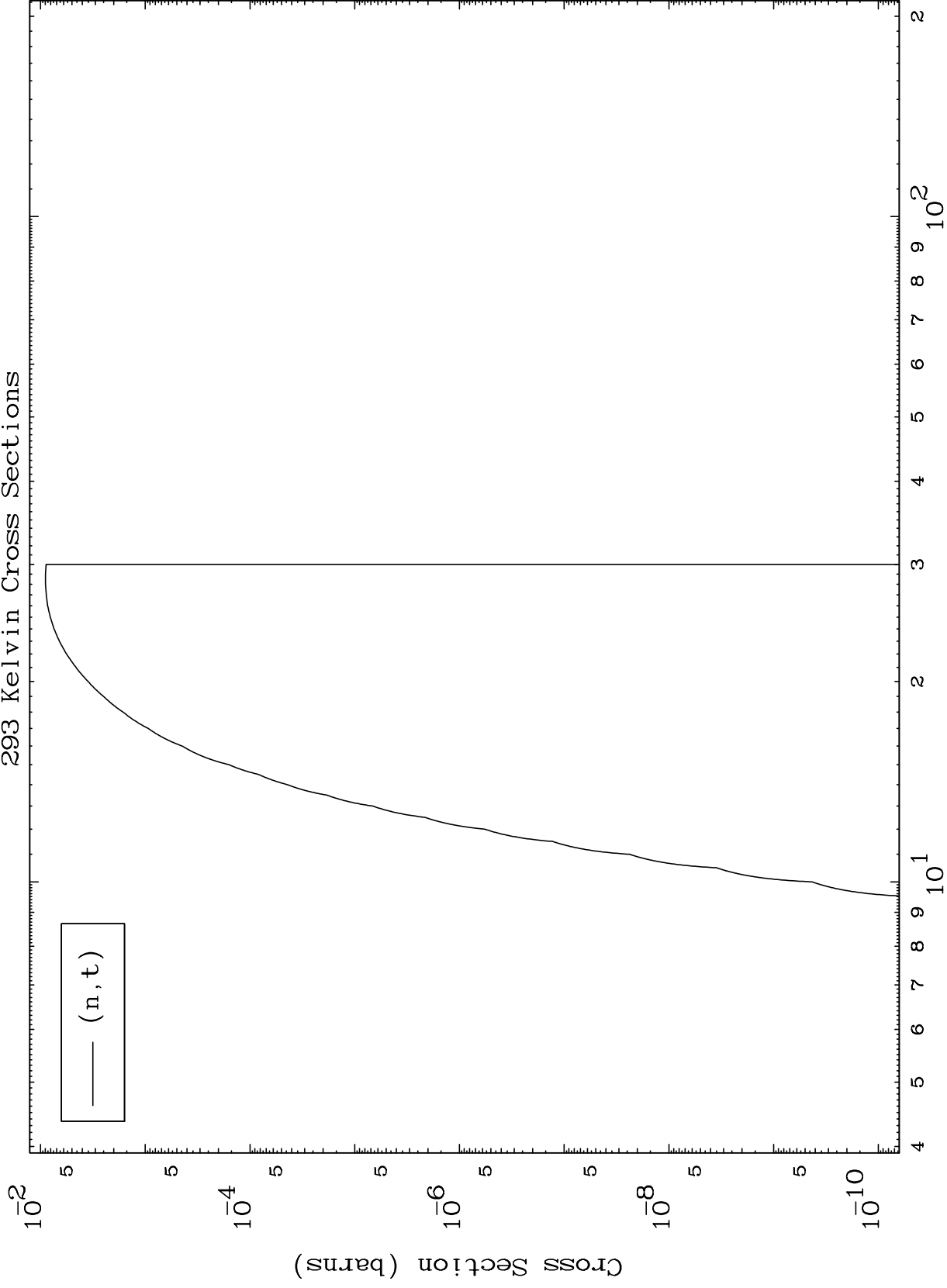
71-Lu-167

Incident Energy (MeV)

MAT 7101

(n,t) Levels
293 Kelvin Cross Sections

71-Lu-167



11

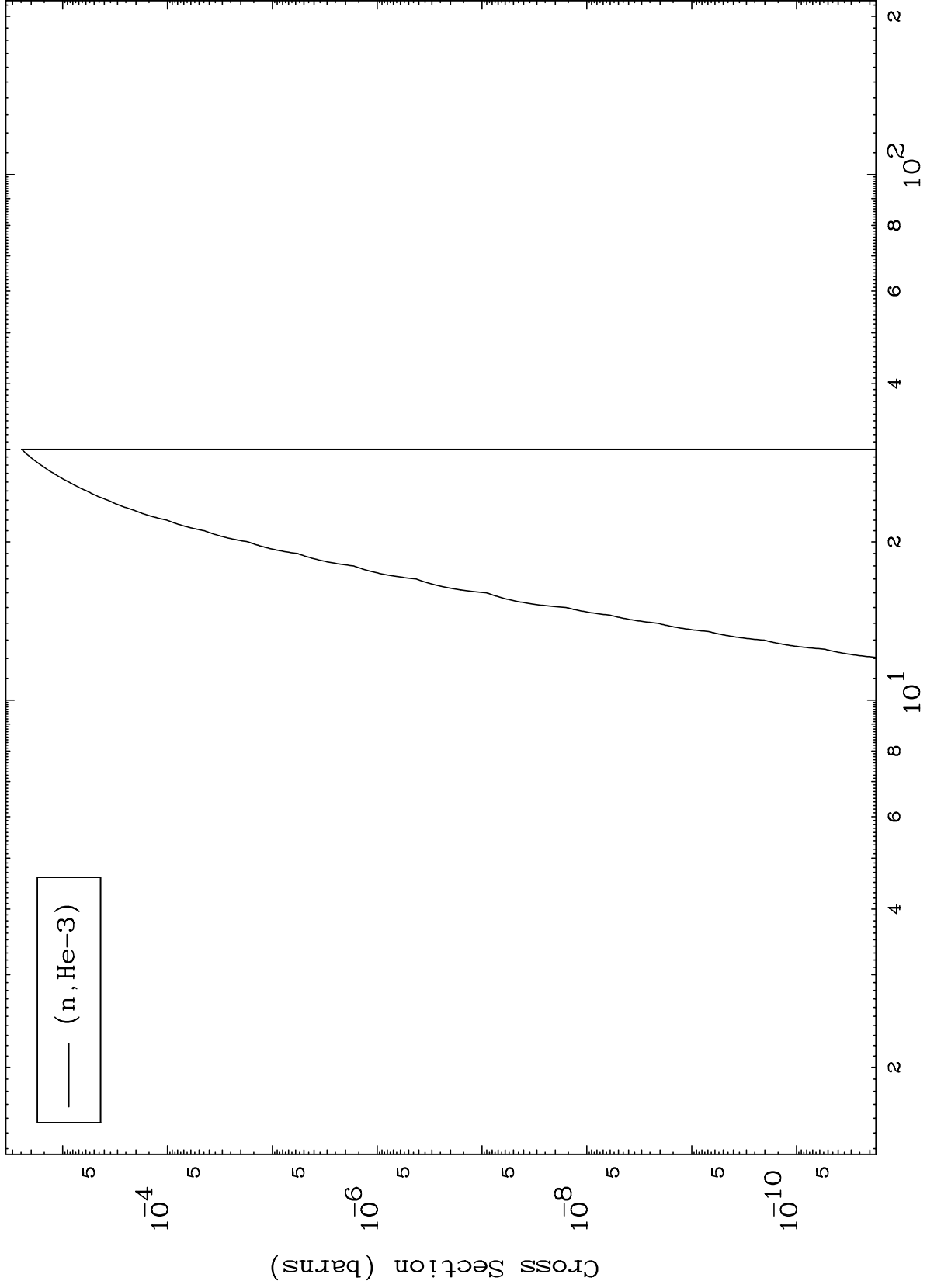
Incident Energy (MeV)

71-Lu-167

MAT 7101

(n,He3) Levels
293 Kelvin Cross Sections

71-Lu-167



12

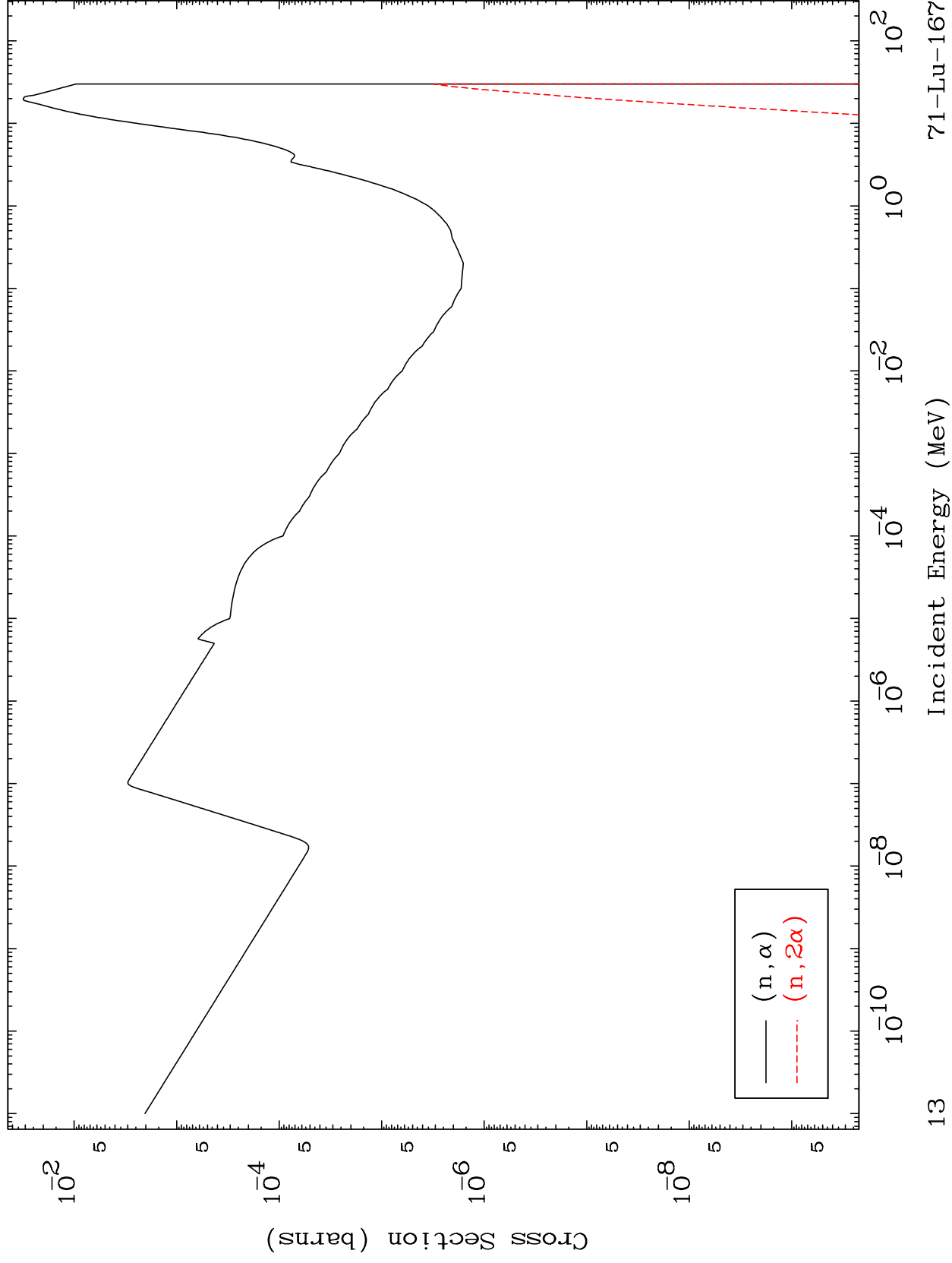
Incident Energy (MeV)

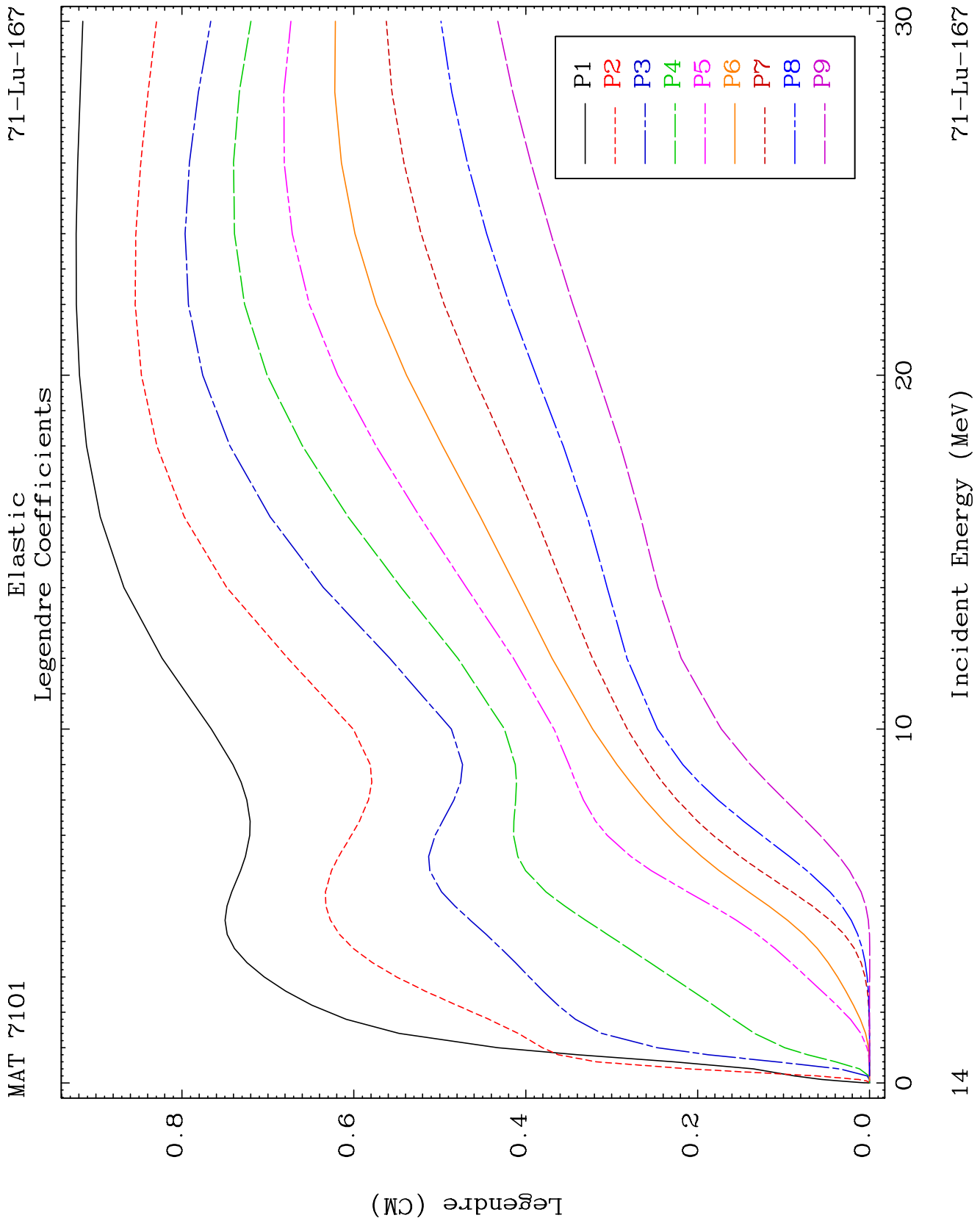
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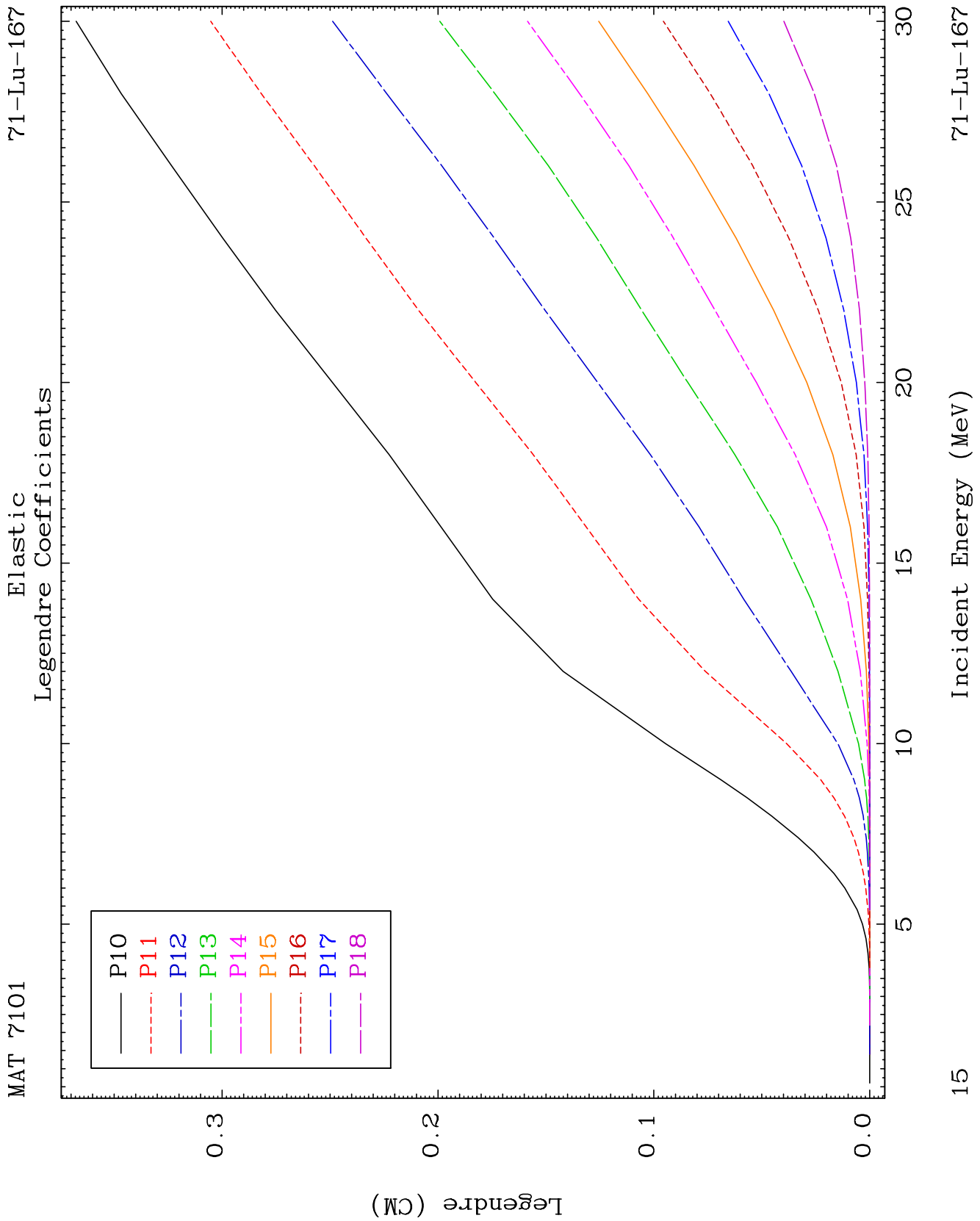
MAT 7101

(n,α) Levels
293 Kelvin Cross Sections

71-Lu-167



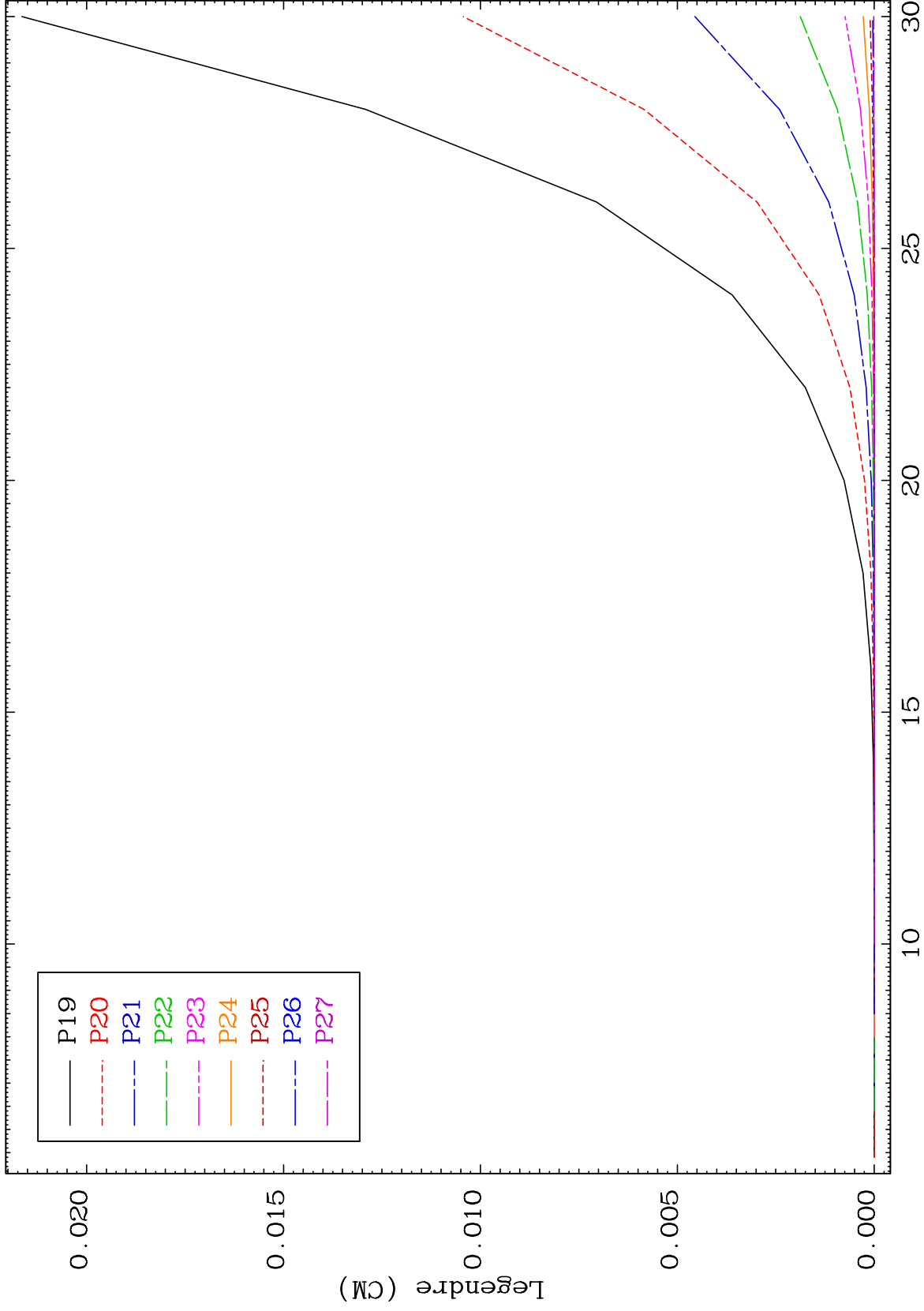




MAT 7101

Elastic Legendre Coefficients

71-Lu-167



16

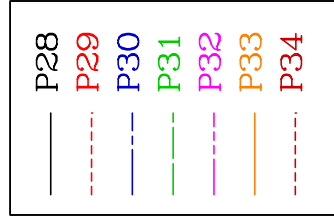
Incident Energy (MeV)

71-Lu-167

MAT 7101

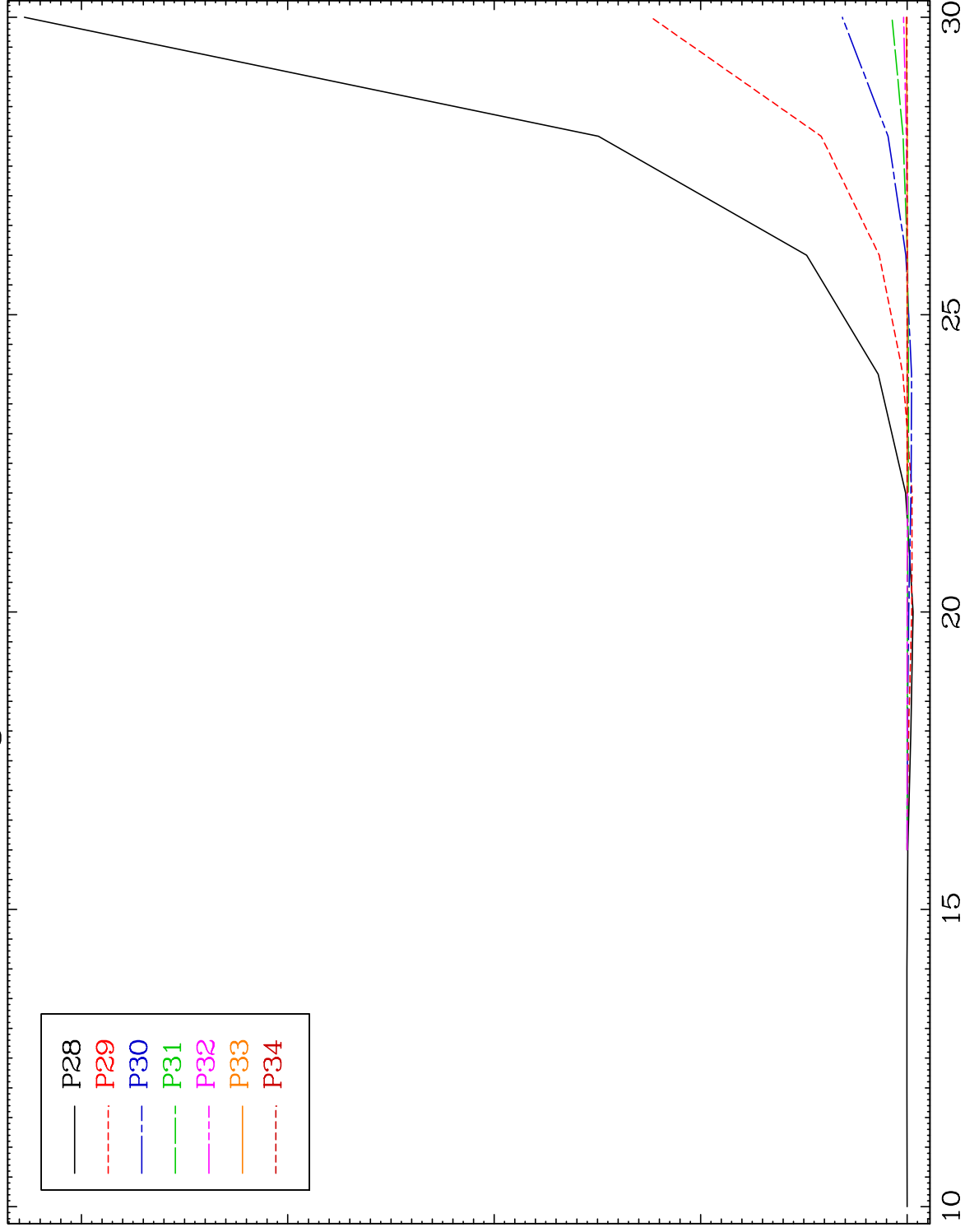
Elastic
Legendre Coefficients

71-Lu-167



$\times 10^{-6}$

Legendre (CM)



10

15

20

25

30

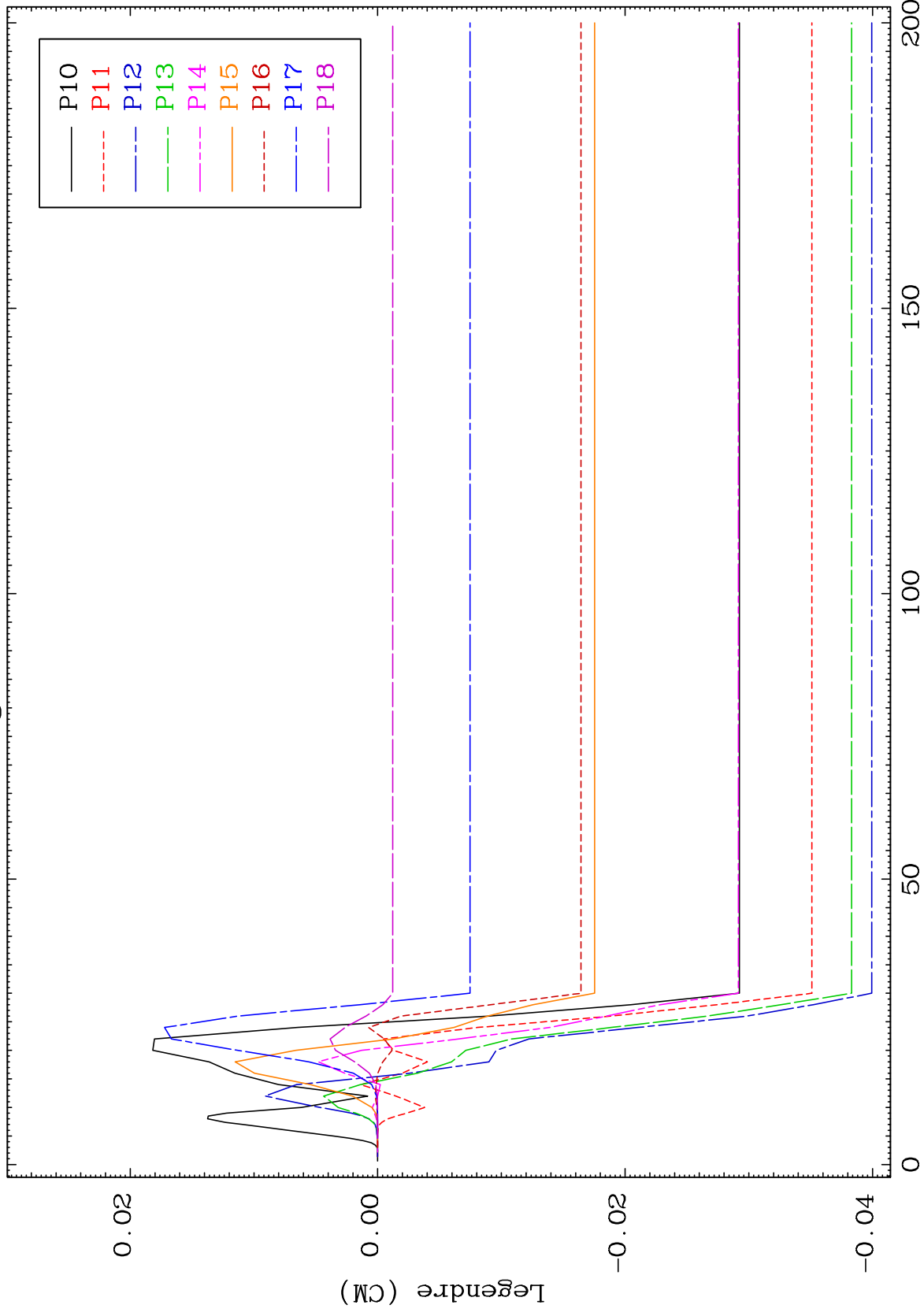
Incident Energy (MeV)

71-Lu-167

MAT 7101

MT= 51 (n,n') Level
Legendre Coefficients

71-Lu-167



19

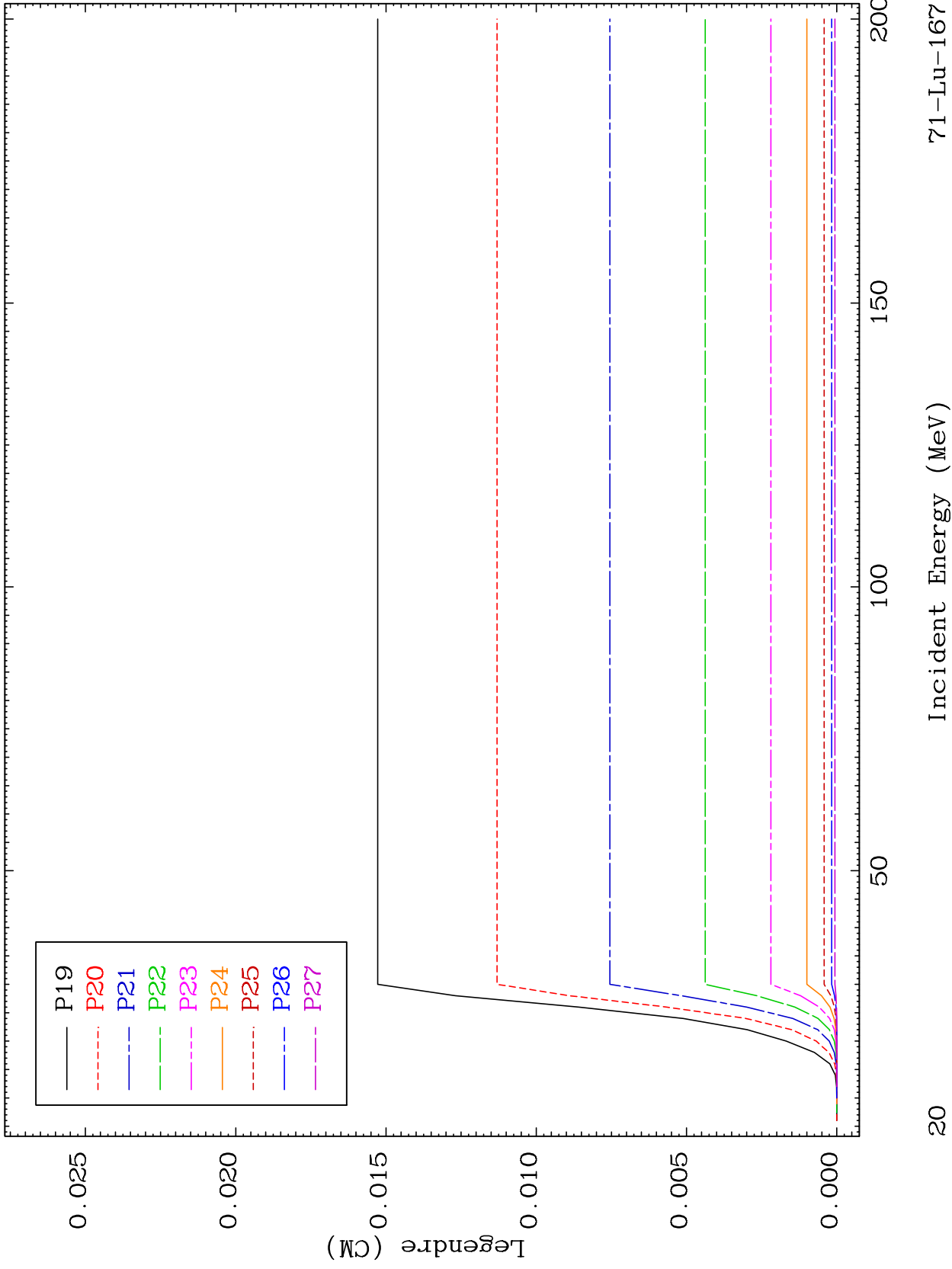
Incident Energy (MeV)

71-Lu-167

MAT 7101

MT= 51 (n,n') Level
Legendre Coefficients

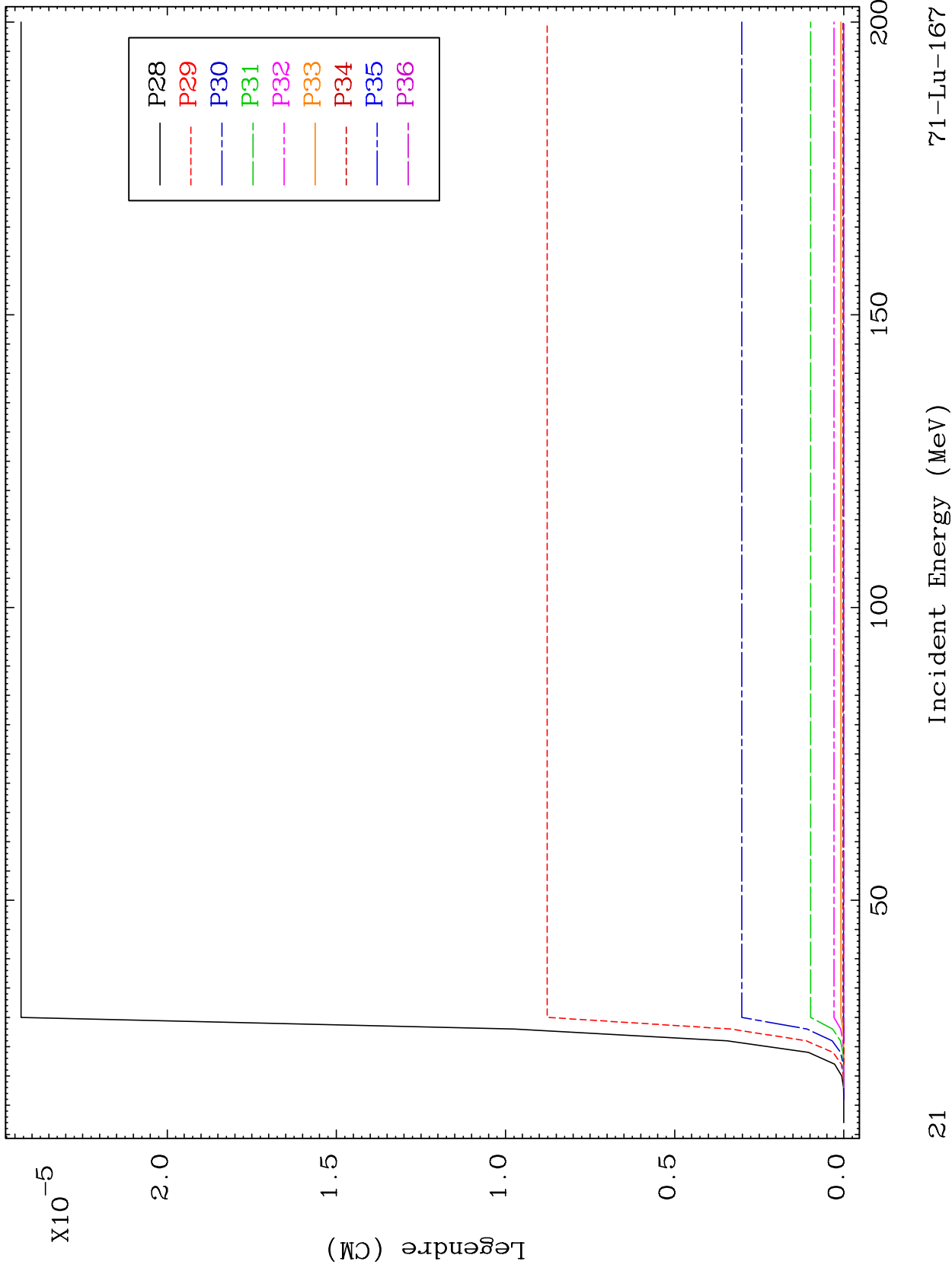
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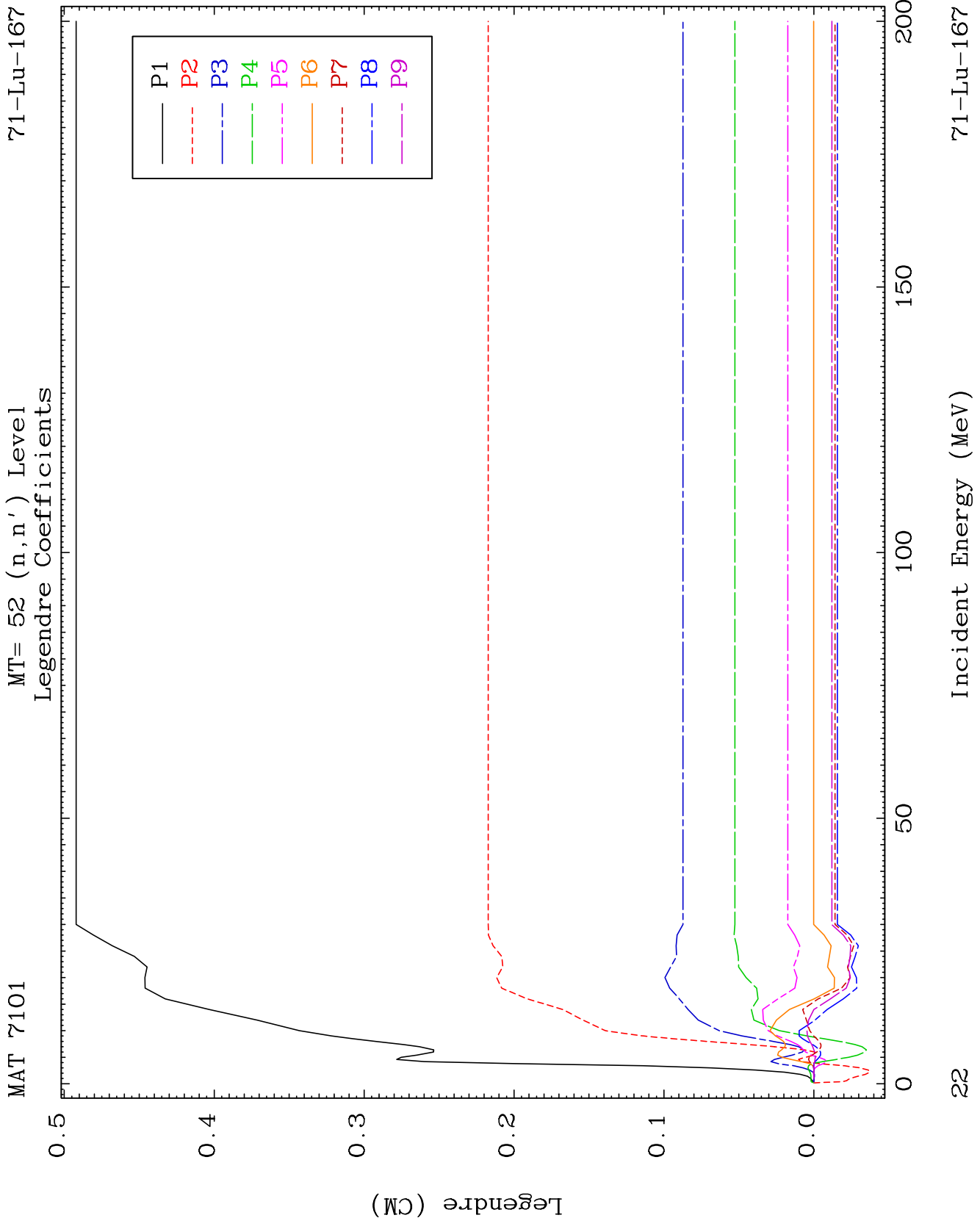


71-Lu-167

Incident Energy (MeV)

20

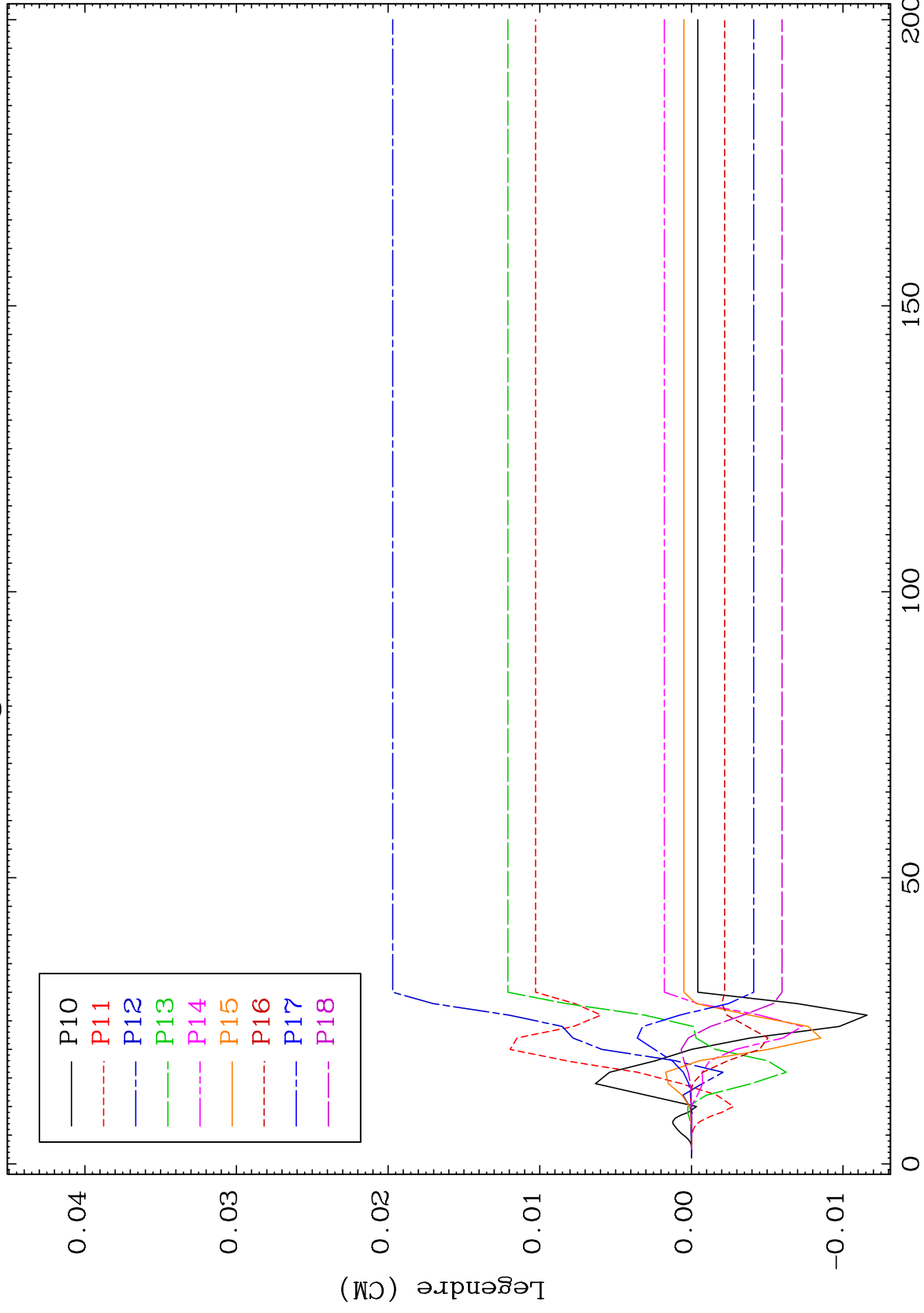




MAT 7101

MT= 52 (n,n') Level
Legendre Coefficients

71-Lu-167



23

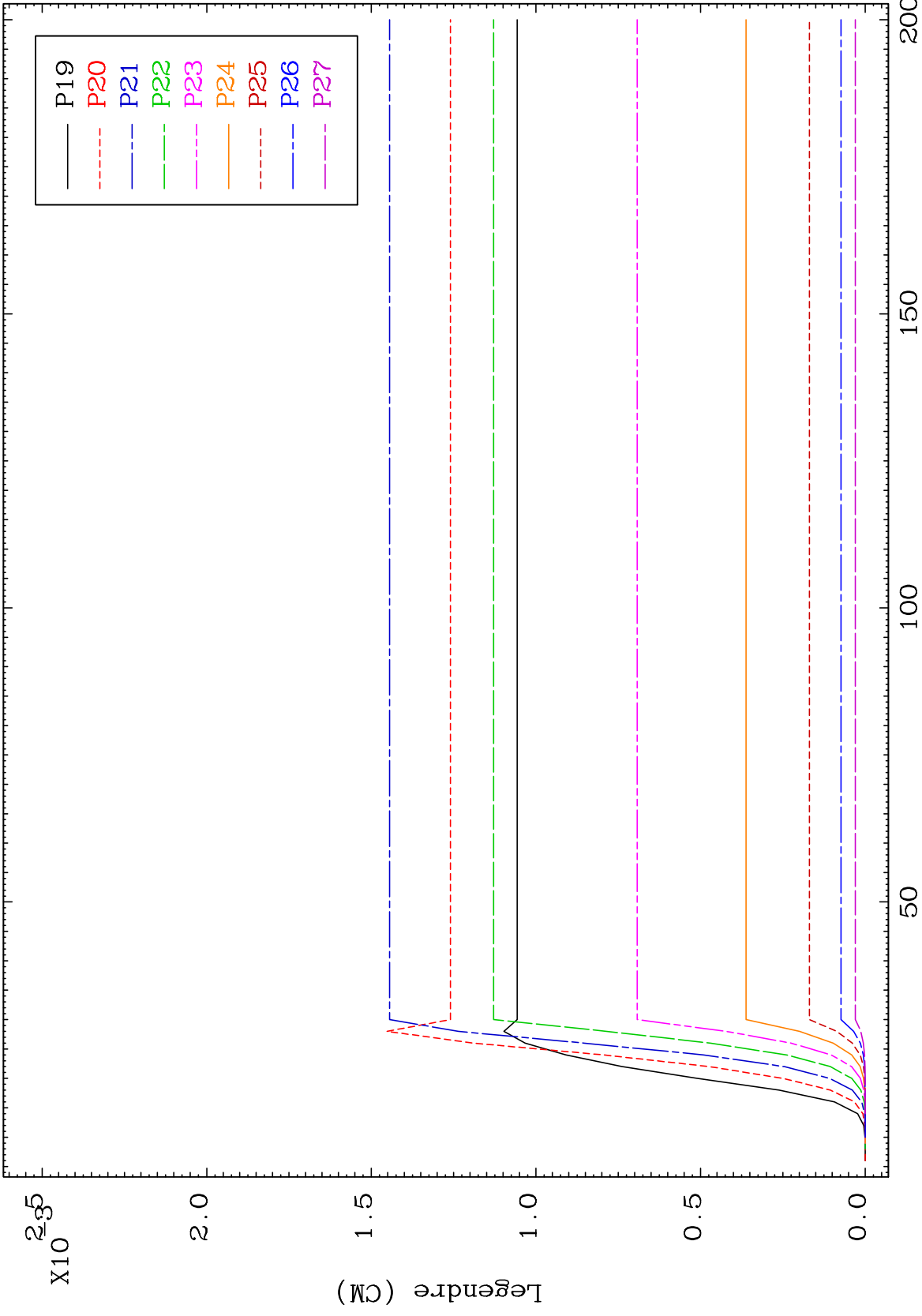
Incident Energy (MeV)

71-Lu-167

MAT 7101

MT= 52 (n,n') Level
Legendre Coefficients

71-Lu-167



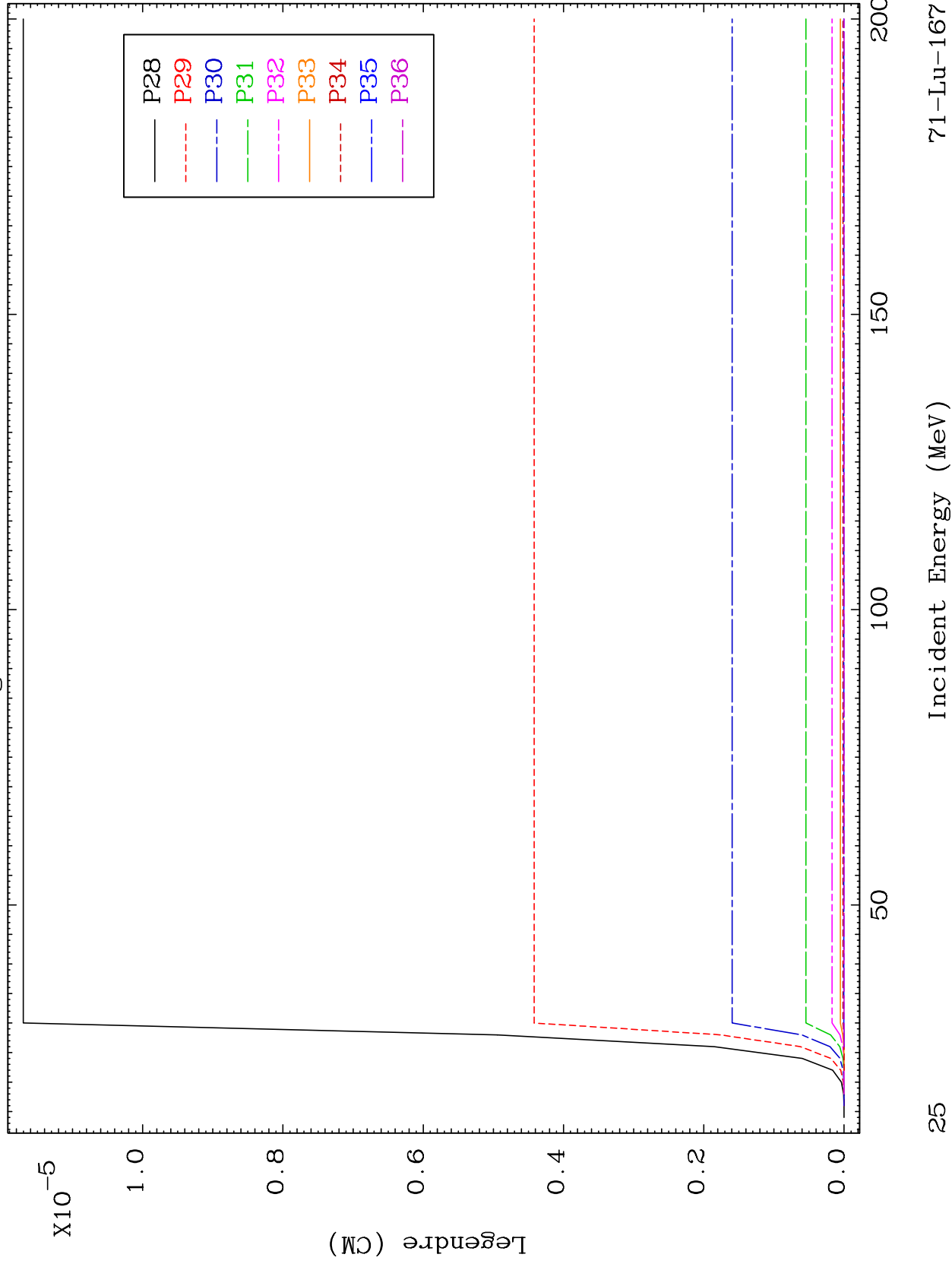
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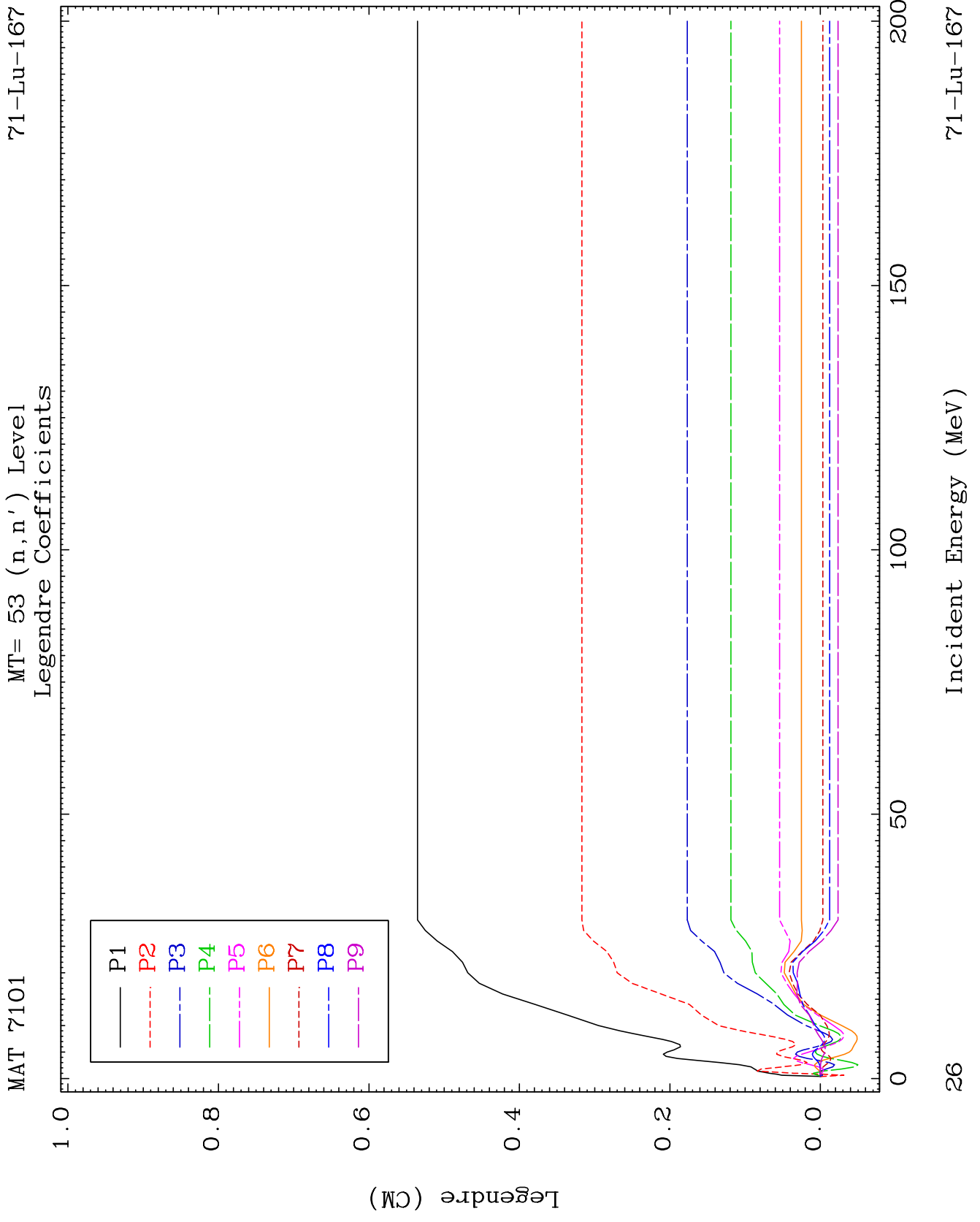
71-Lu-167

MAT 7101

MT= 52 (n,n') Level
Legendre Coefficients

71-Lu-167

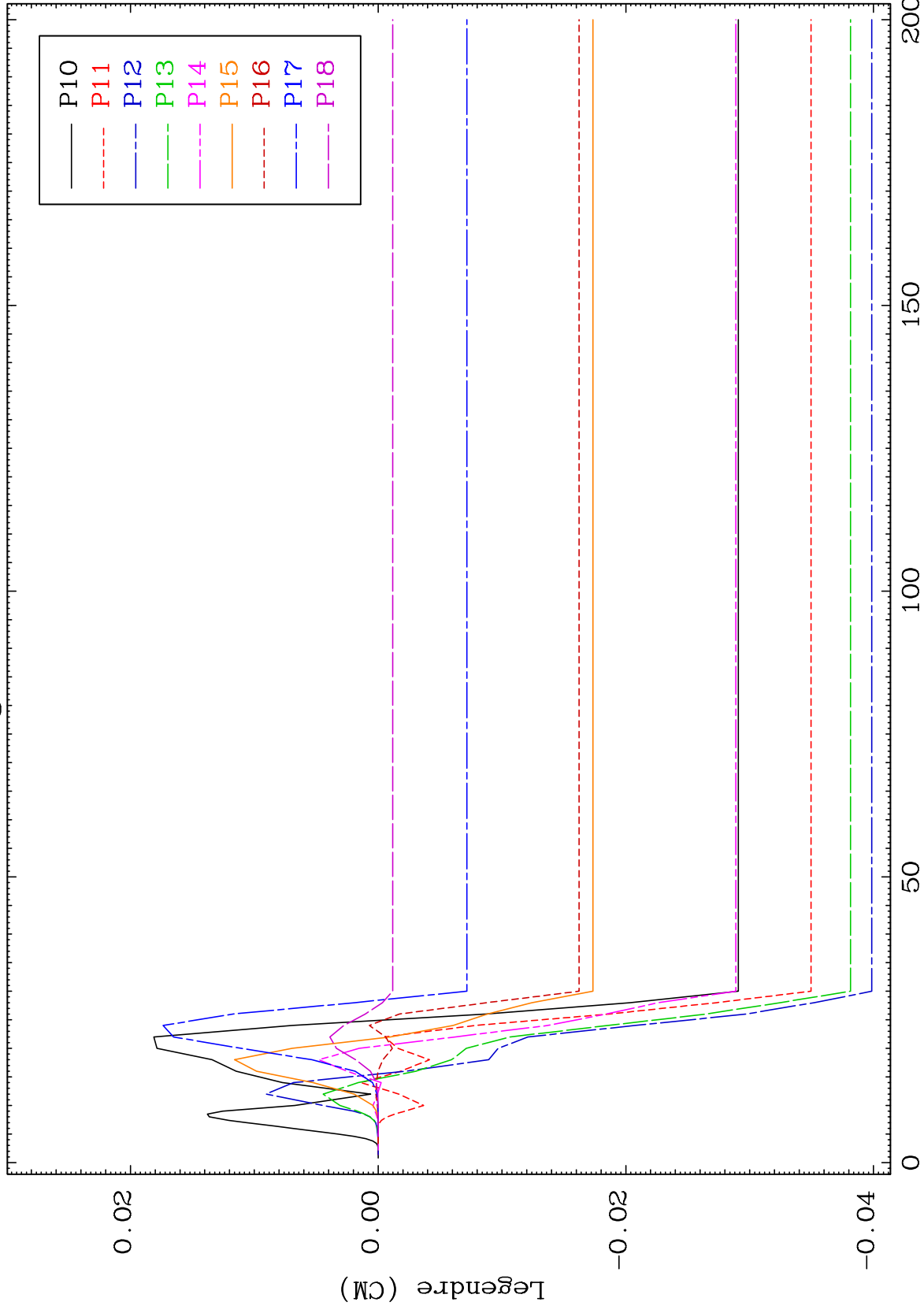




MAT 7101

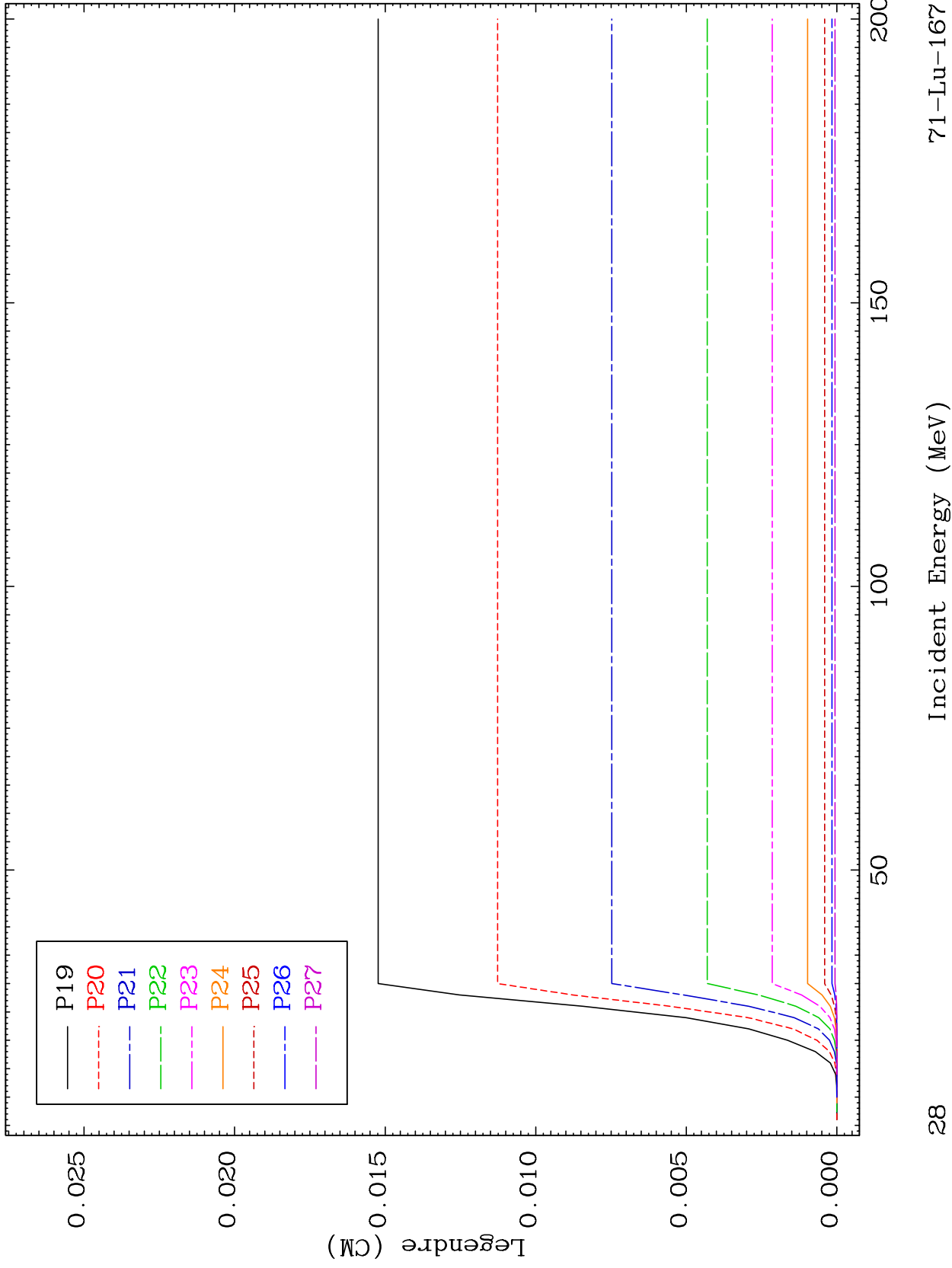
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Legendre Coefficients

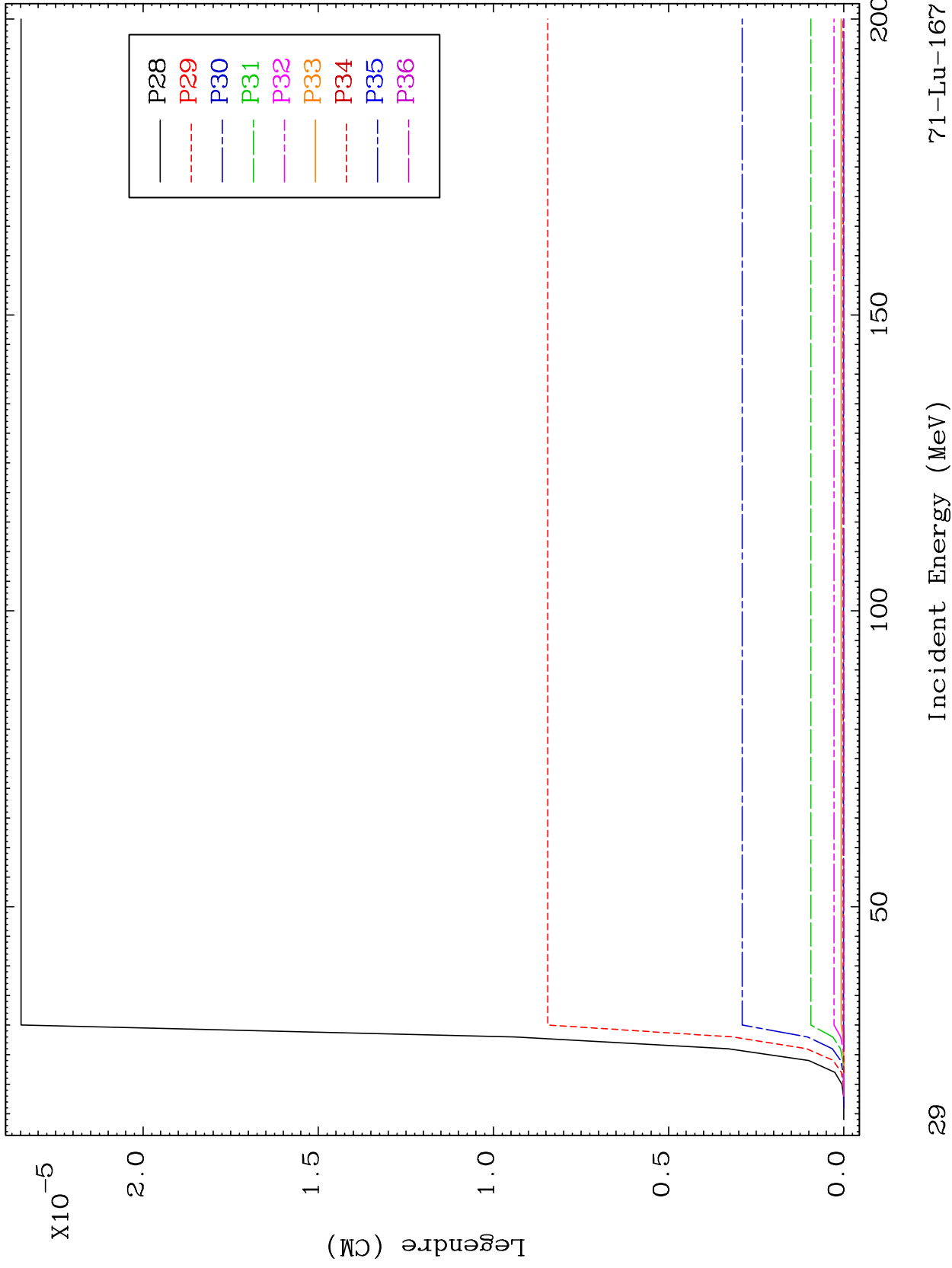
71-Lu-167



27

71-Lu-167

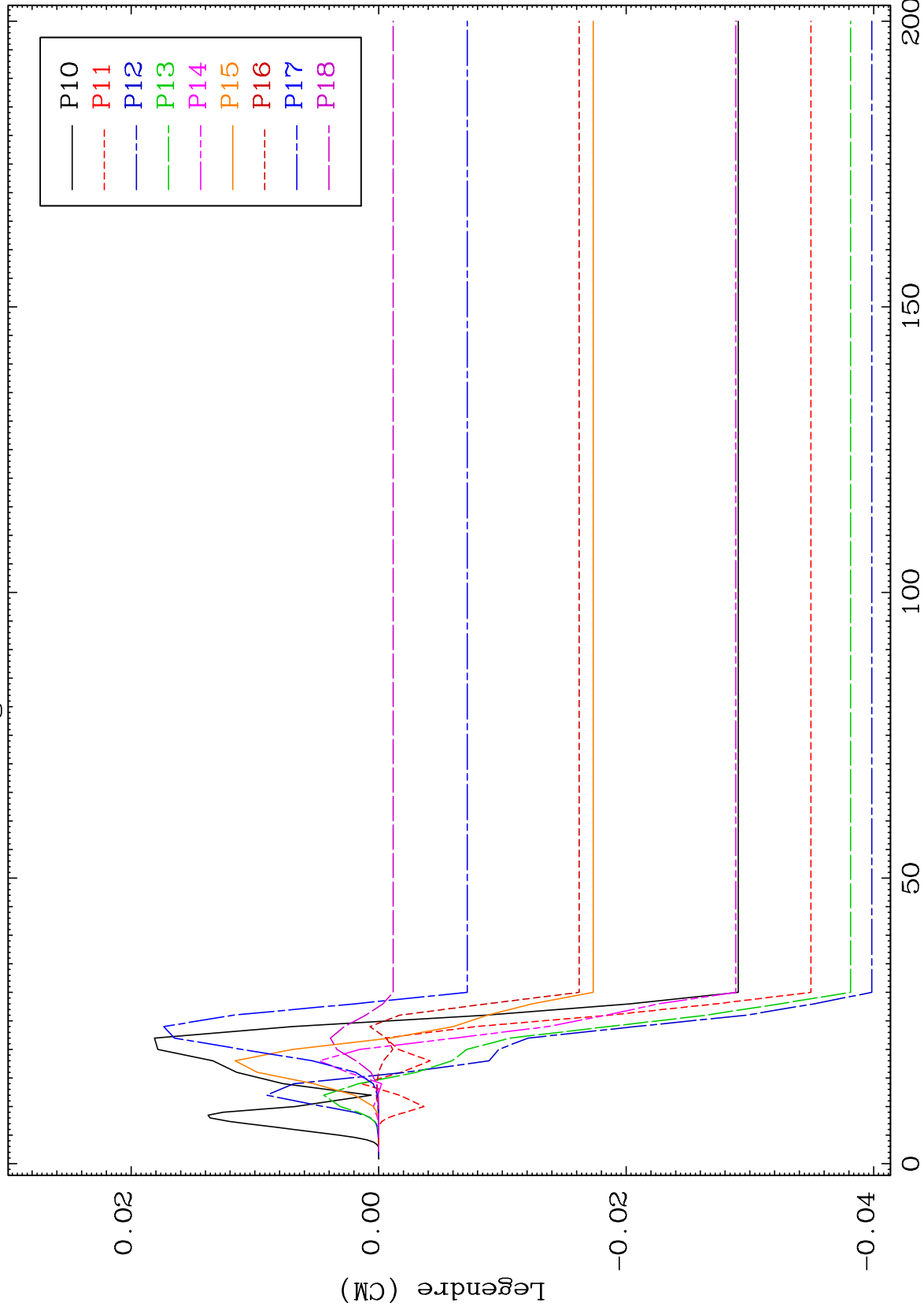




MAT 7101

MT= 54 (n,n') Level
Legendre Coefficients

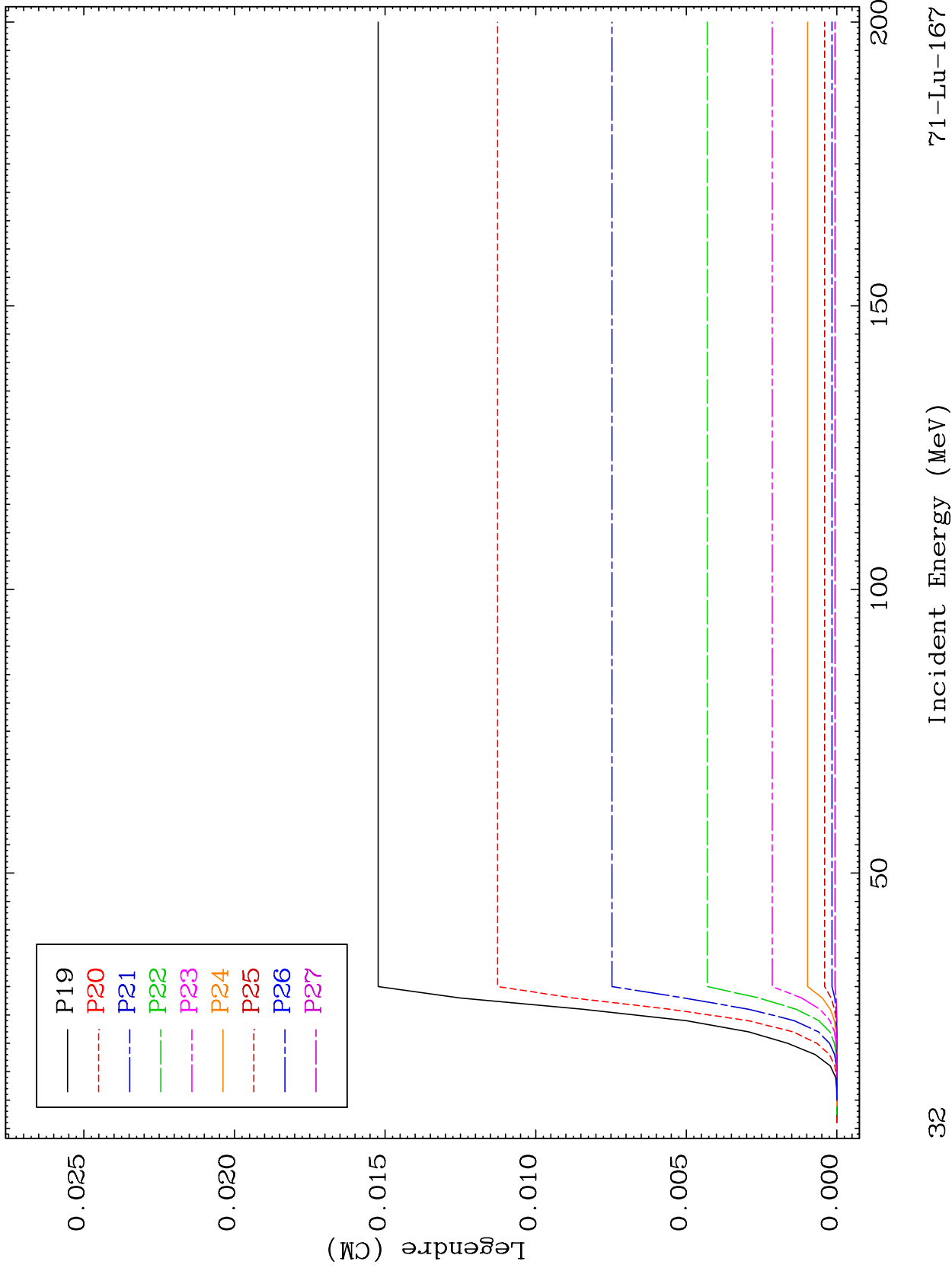
71-Lu-167



31

Incident Energy (MeV)

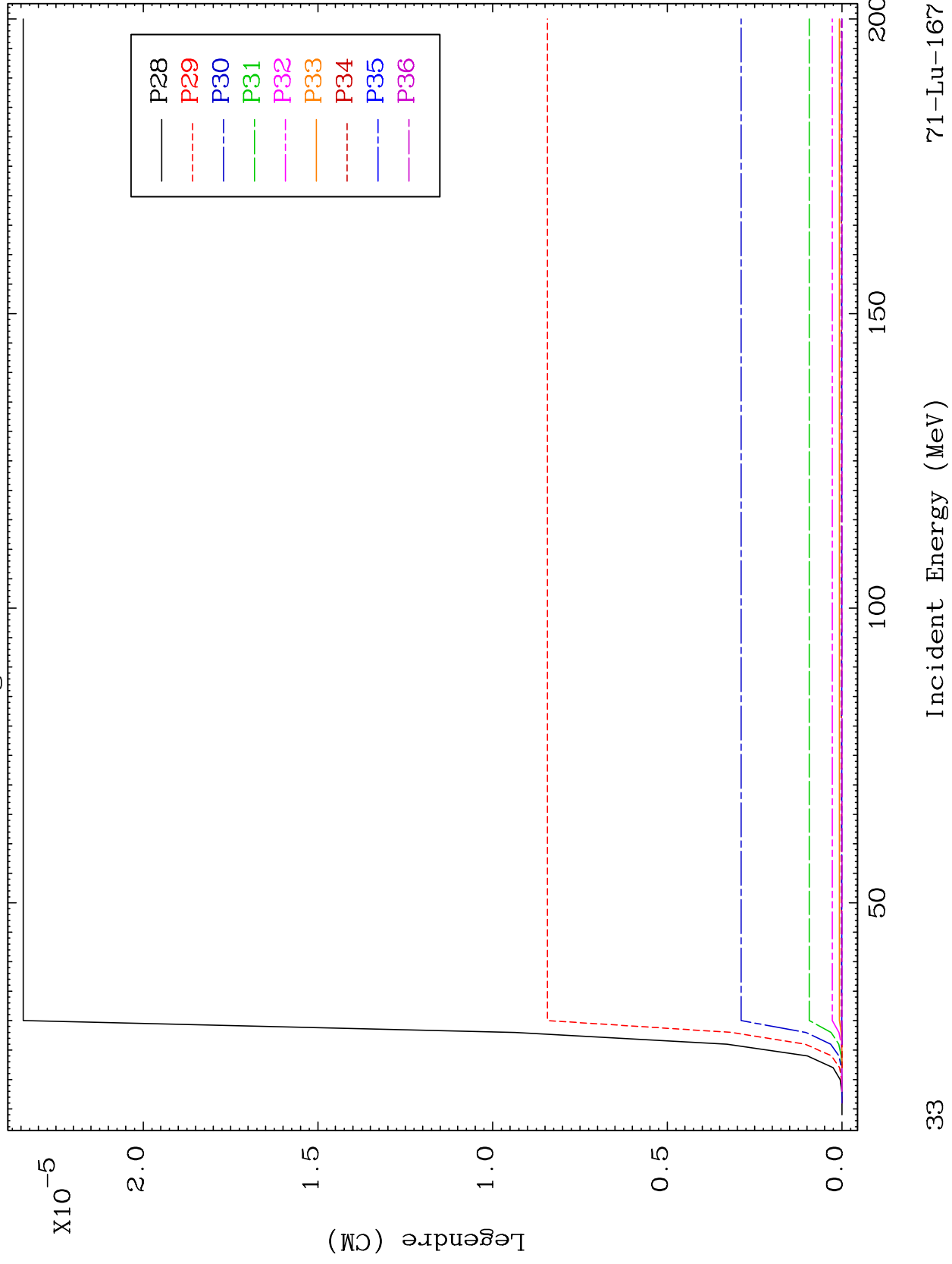
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MAT 7101

MT= 54 (n,n') Level
Legendre Coefficients

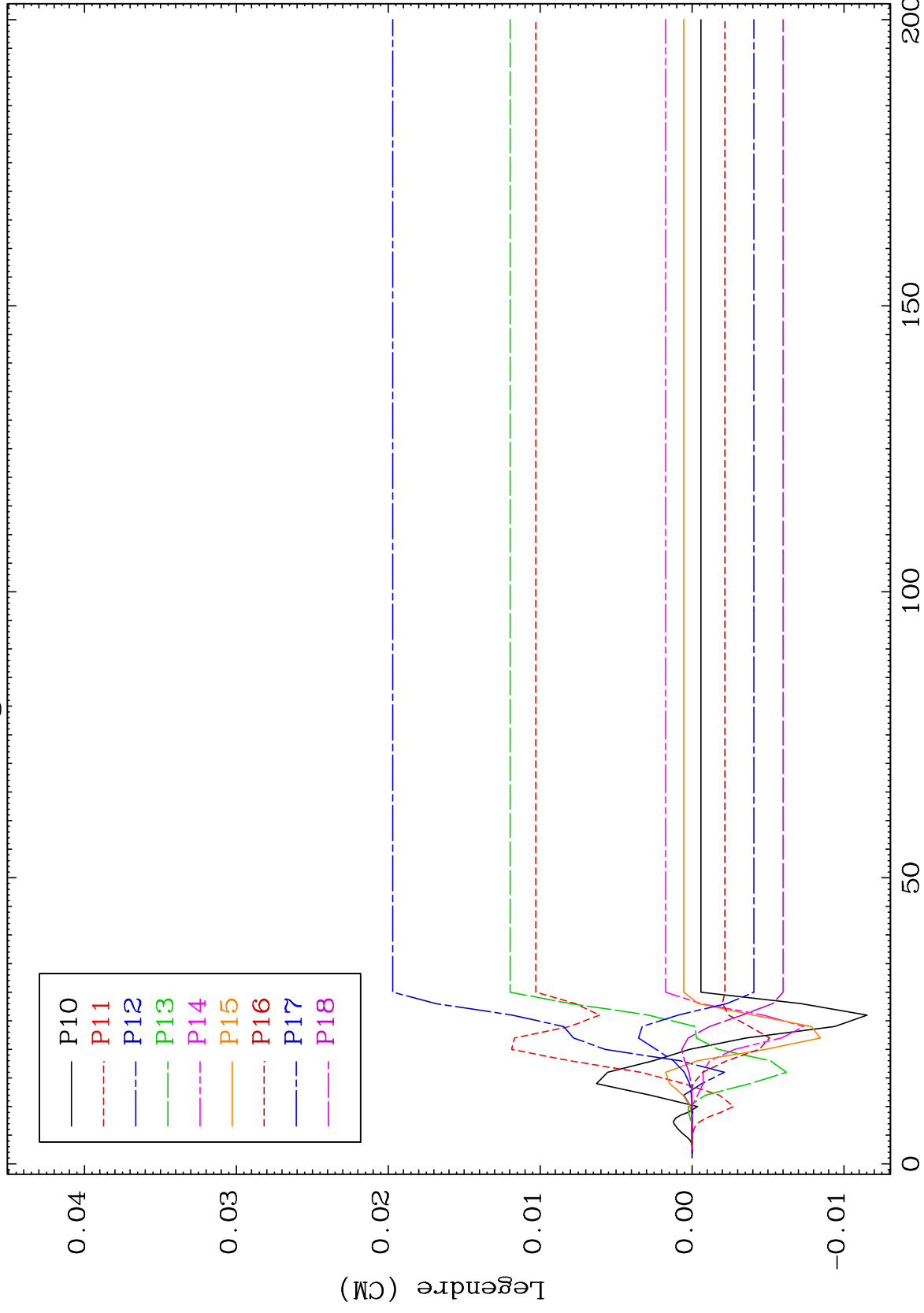
71-Lu-167



MAT 7101

MT= 55 (n,n') Level
Legendre Coefficients

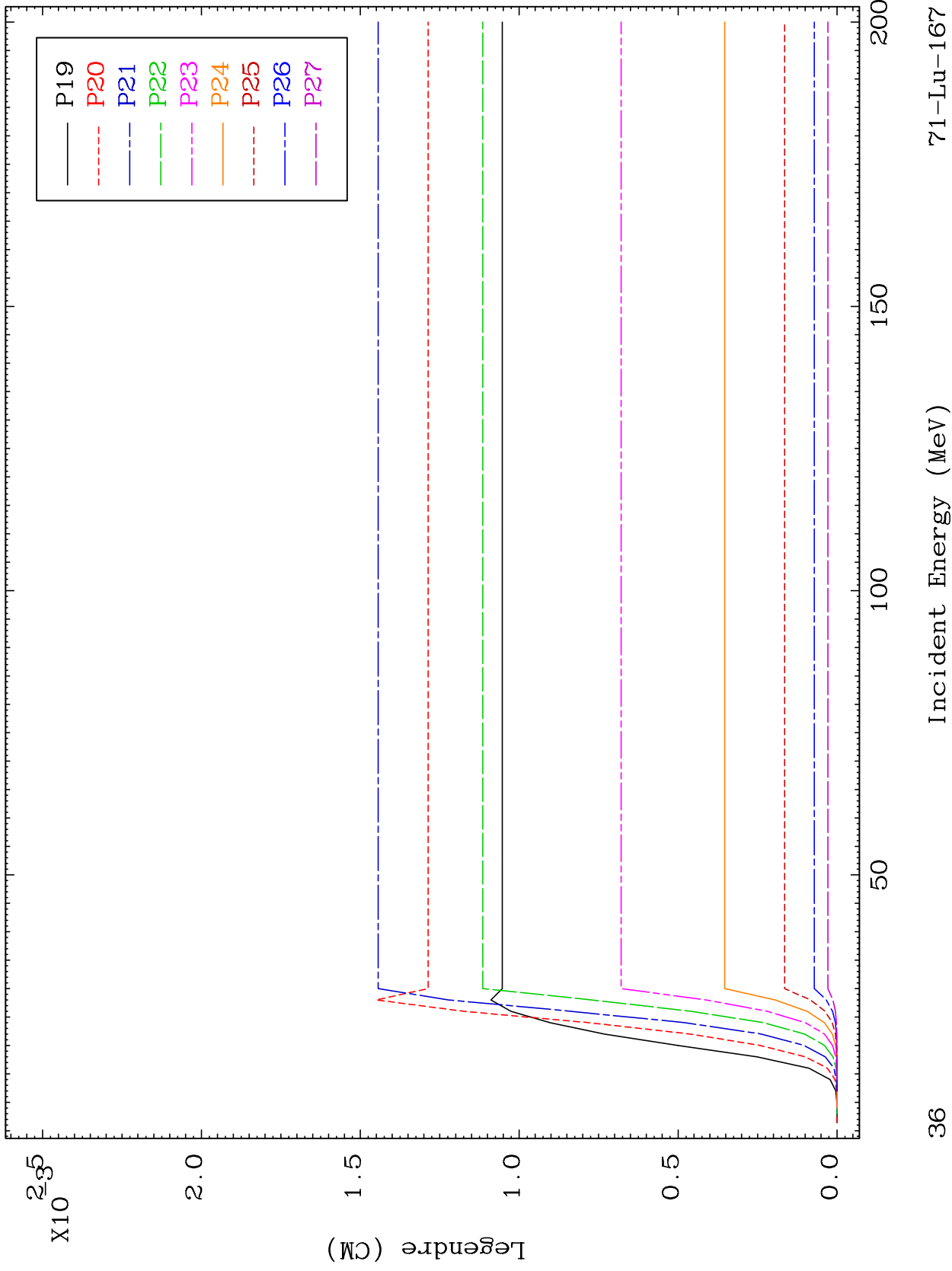
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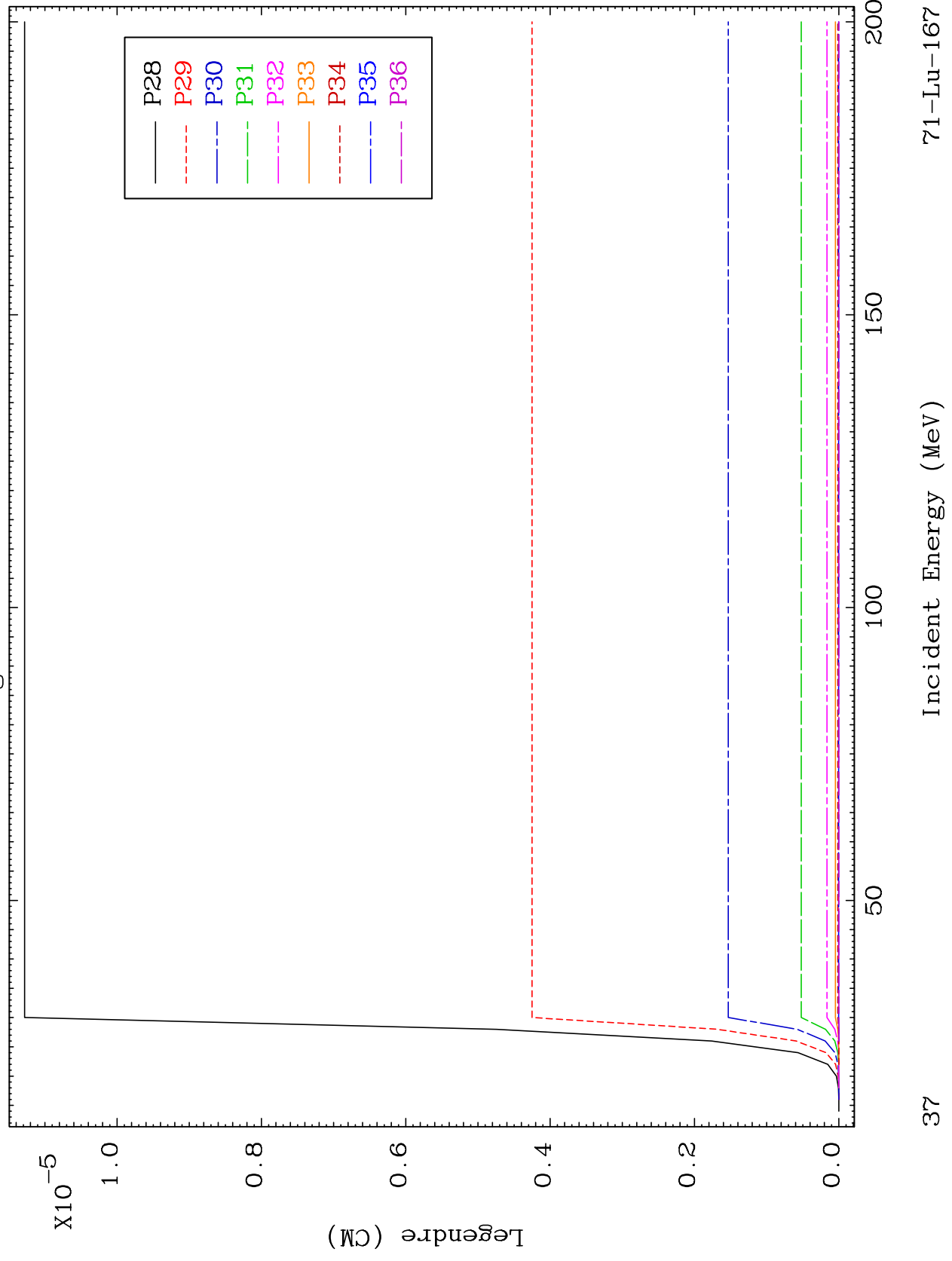


35

Incident Energy (MeV)

71-Lu-167

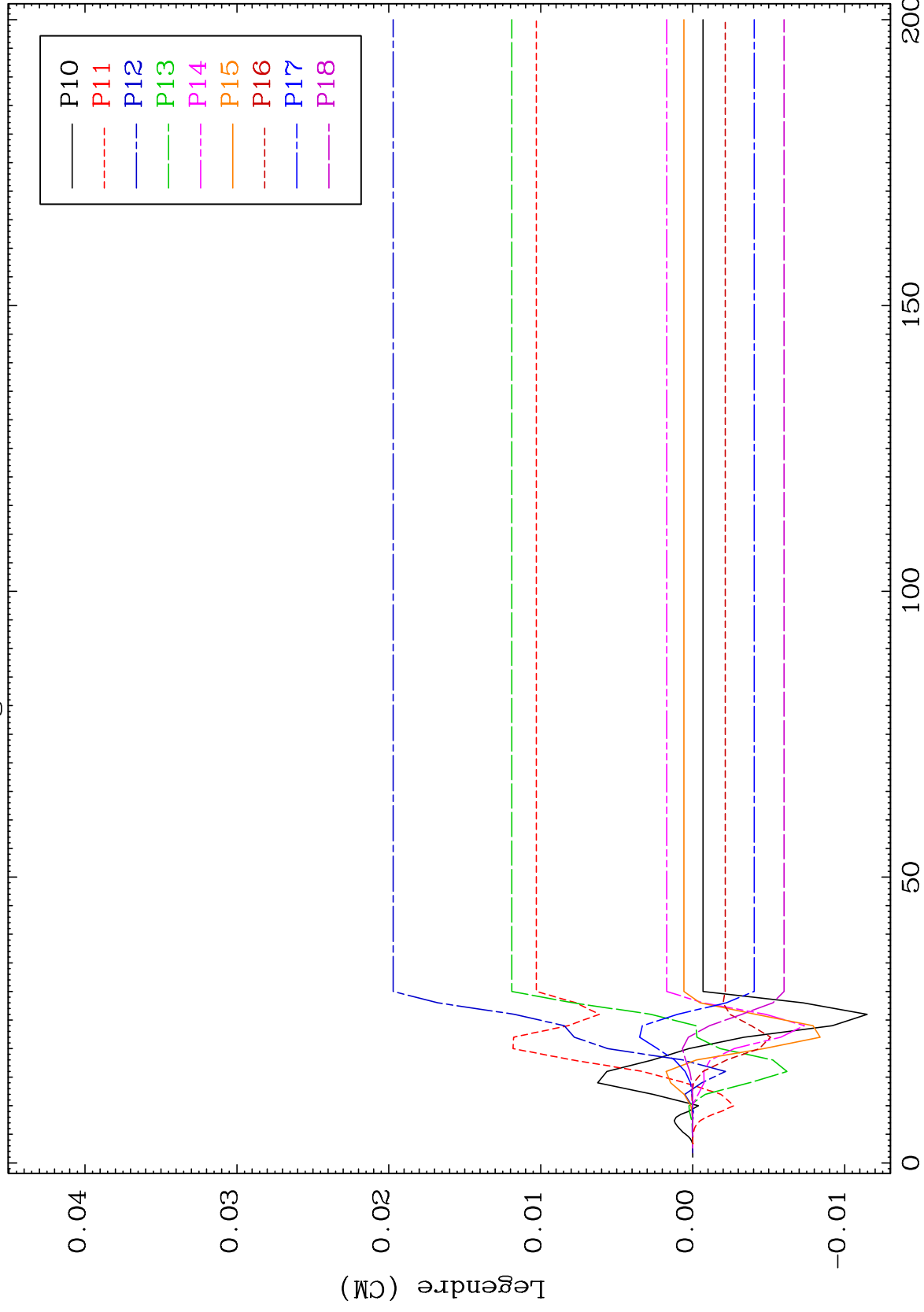




MAT 7101

MT= 56 (n,n') Level
Legendre Coefficients

71-Lu-167



39

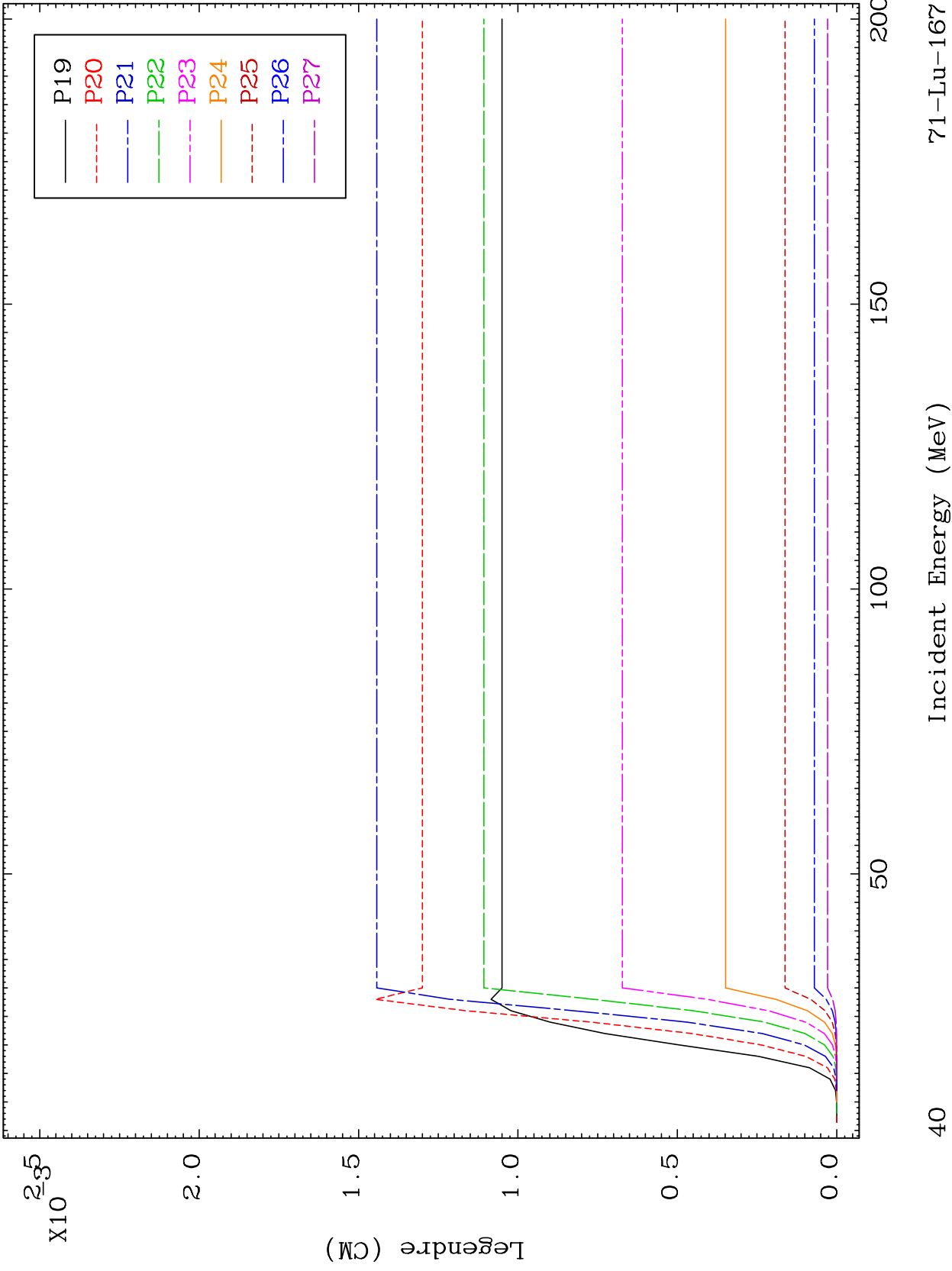
Incident Energy (MeV)

71-Lu-167

MAT 7101

MT= 56 (n,n') Level
Legendre Coefficients

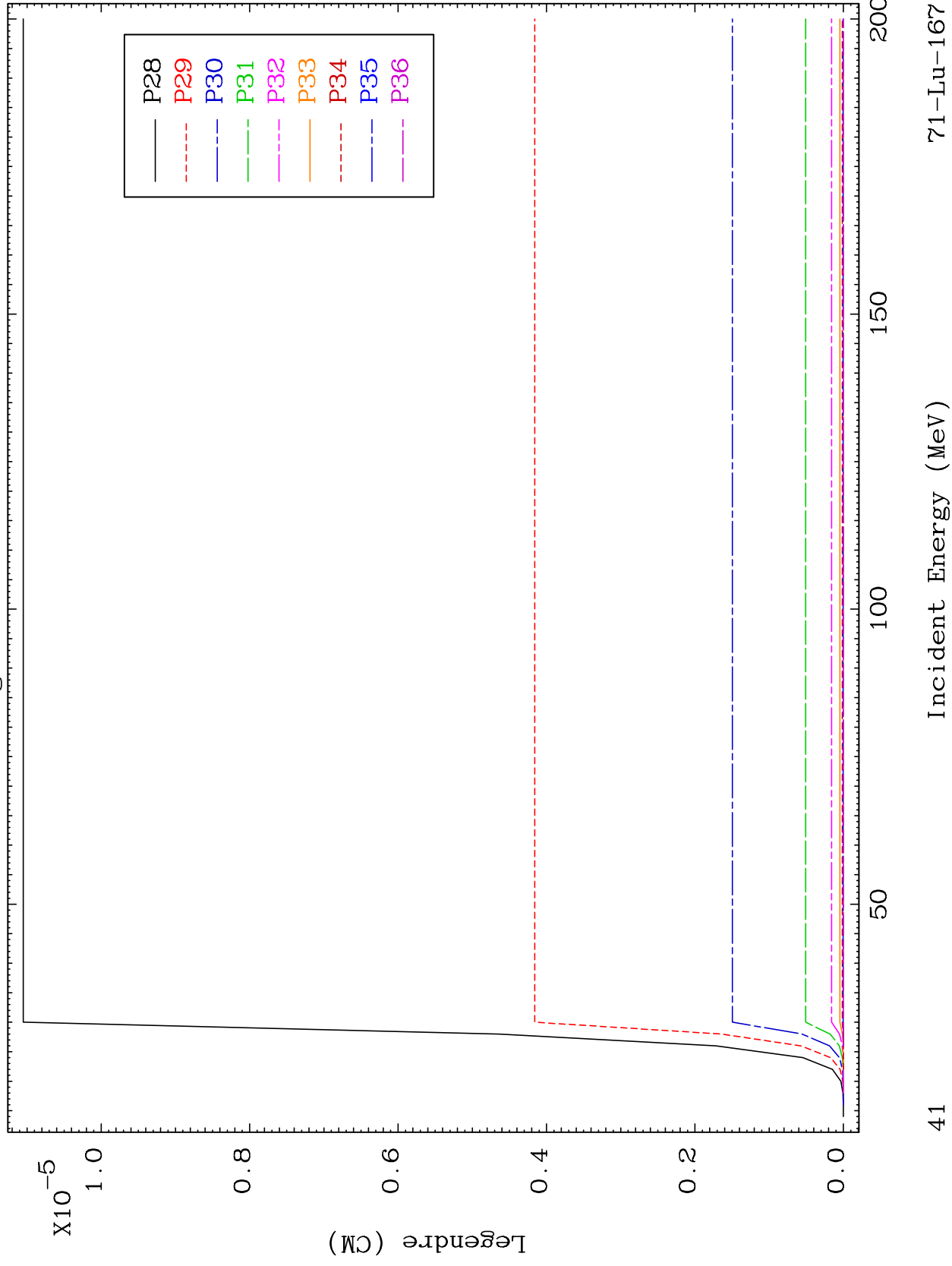
71-Lu-167



MAT 7101

MT= 56 (n,n') Level
Legendre Coefficients

71-Lu-167



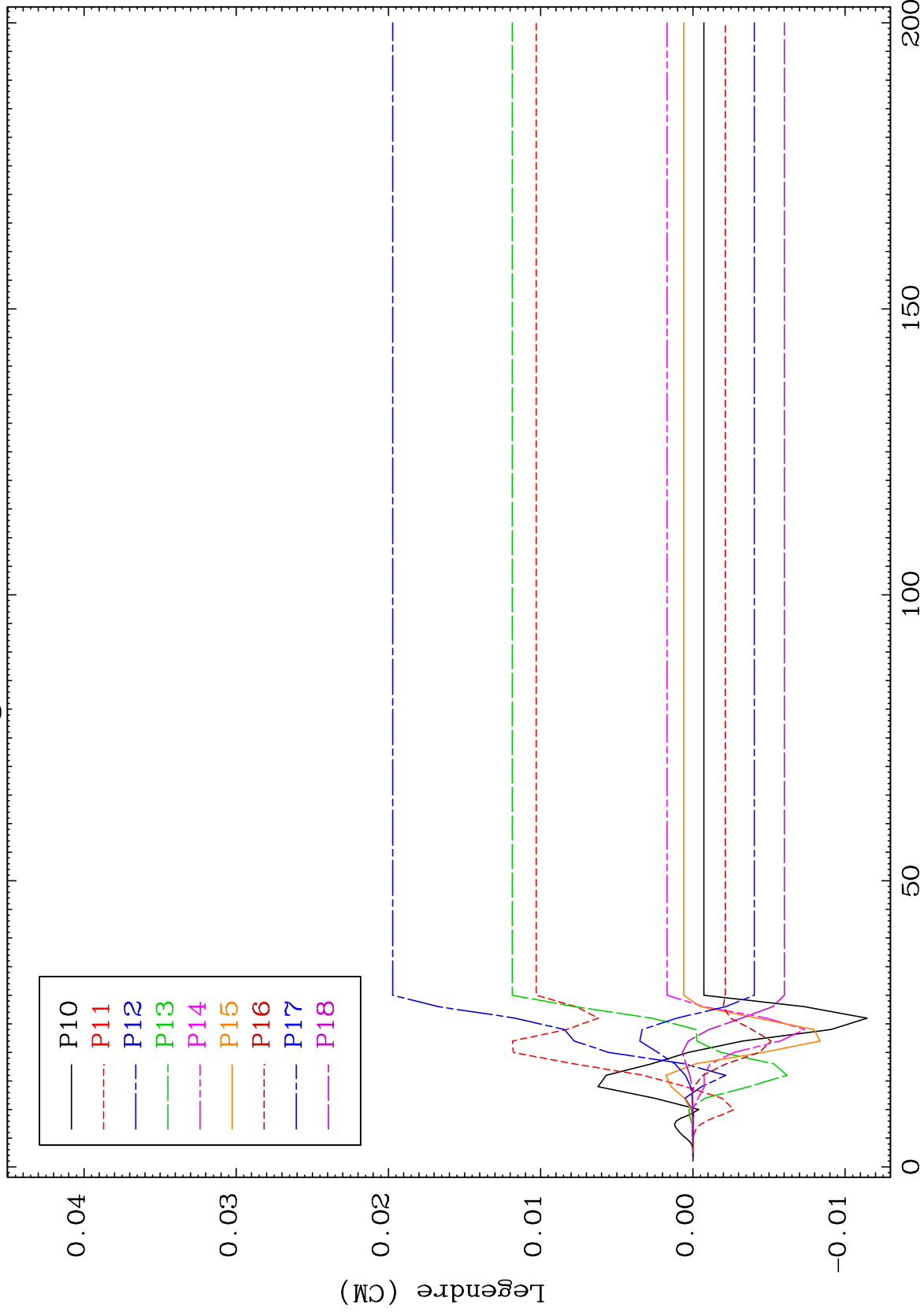
41

71-Lu-167

MAT 7101

MT= 57 (n,n') Level
Legendre Coefficients

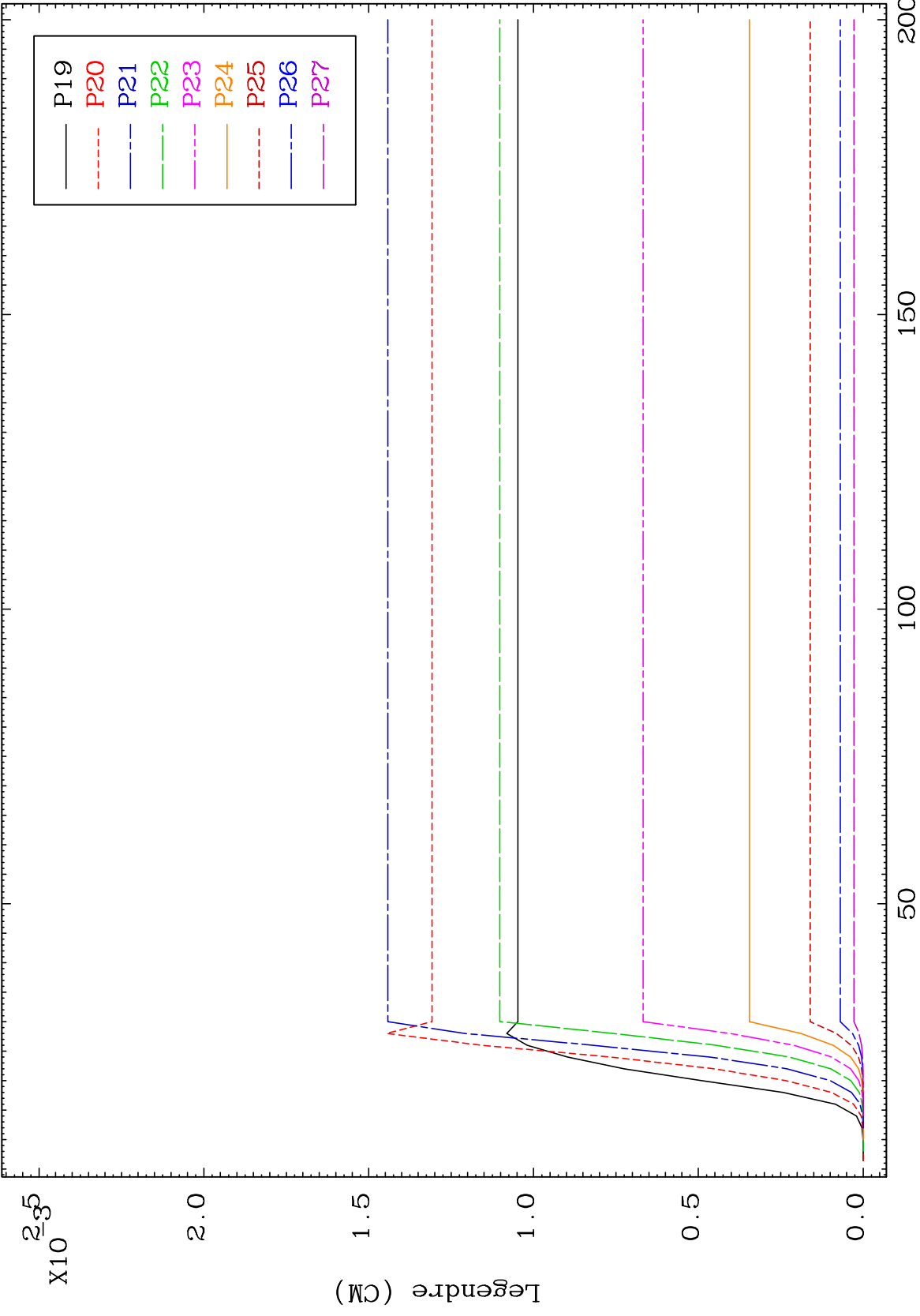
71-Lu-167



43

Incident Energy (MeV)

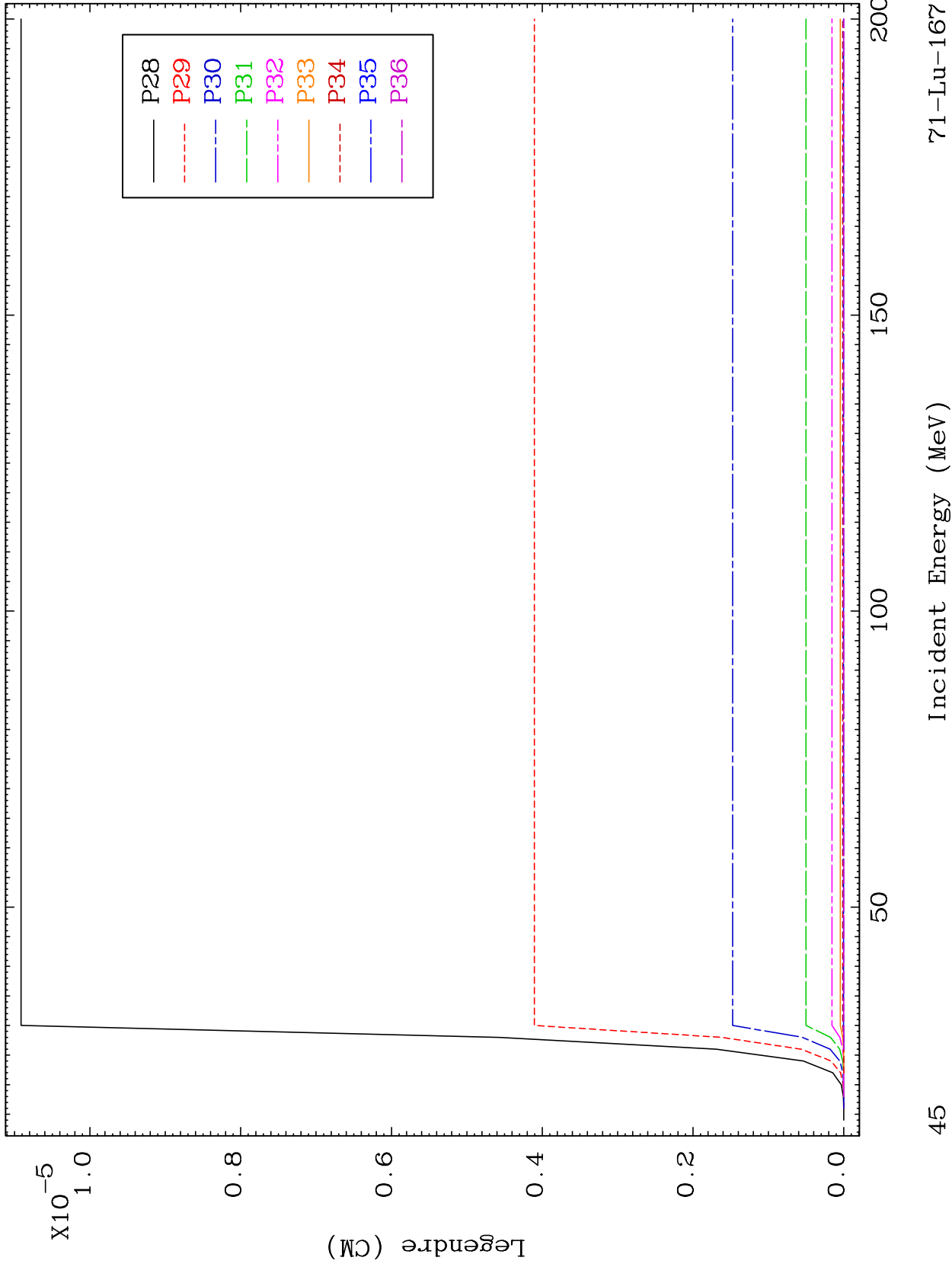
71-Lu-167



MAT 7101

MT= 57 (n,n') Level
Legendre Coefficients

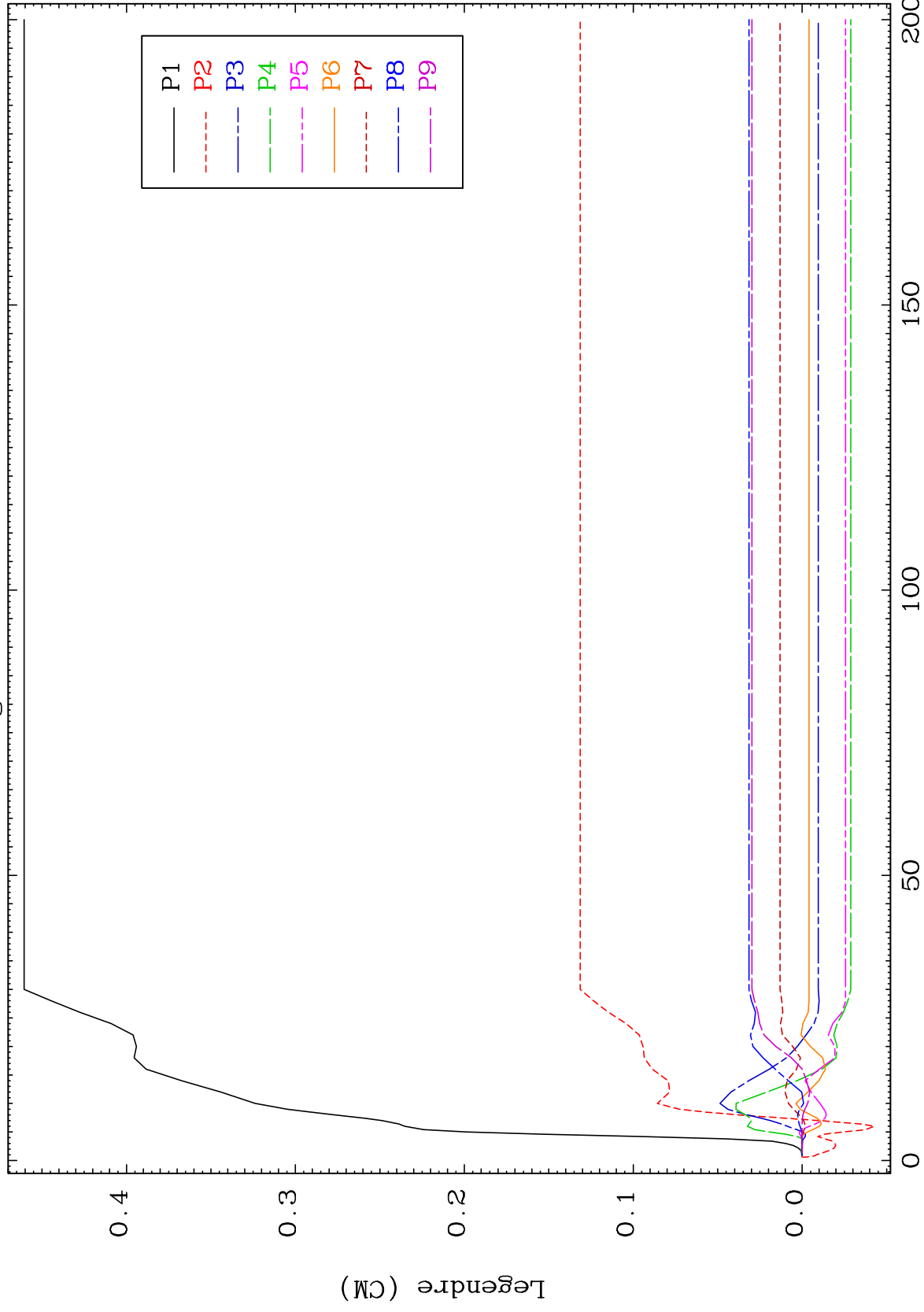
71-Lu-167



MAT 7101

MT= 58 (n,n') Level
Legendre Coefficients

71-Lu-167



46

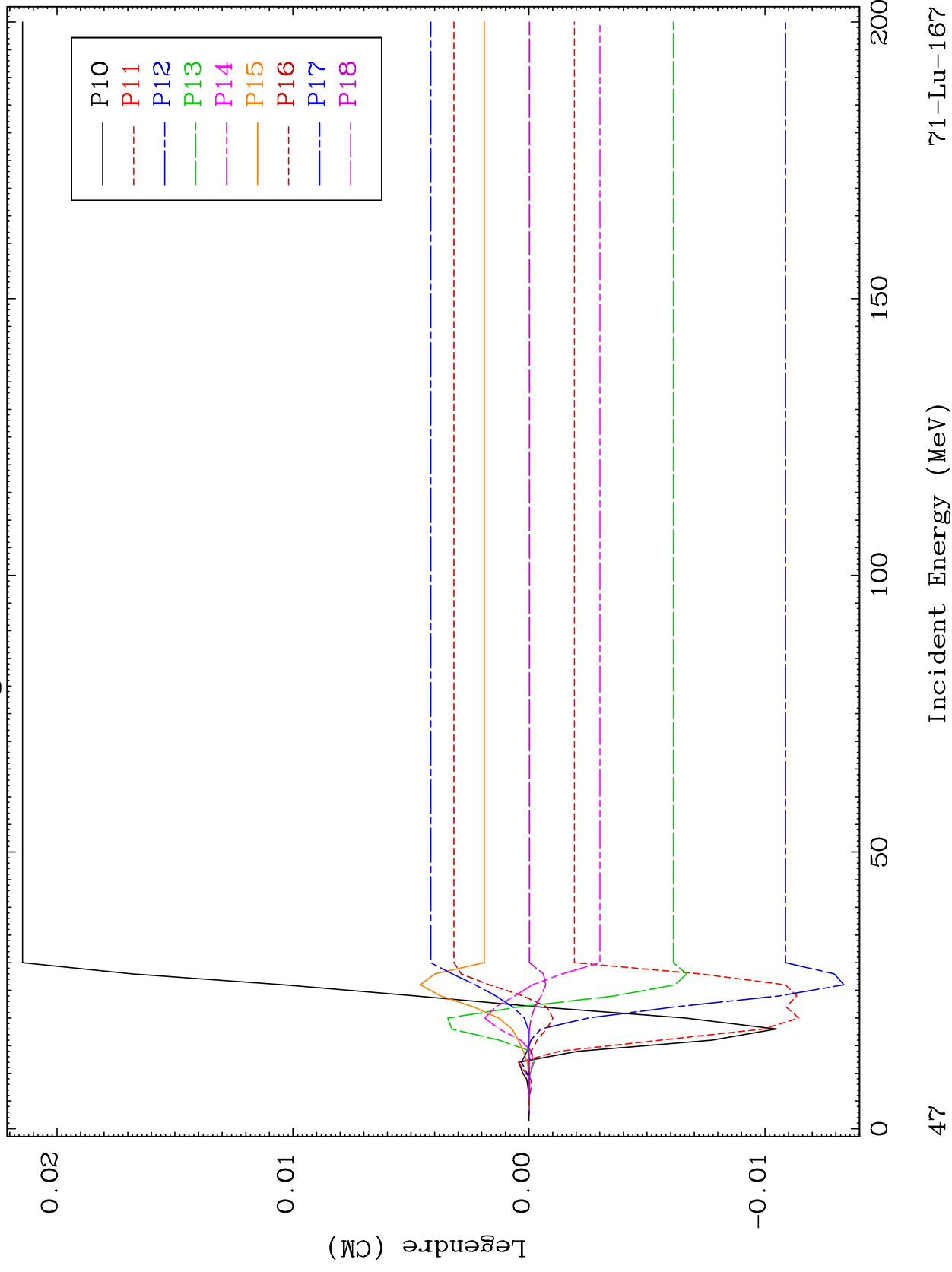
Incident Energy (MeV)

71-Lu-167

MAT 7101

MT= 58 (n,n') Level
Legendre Coefficients

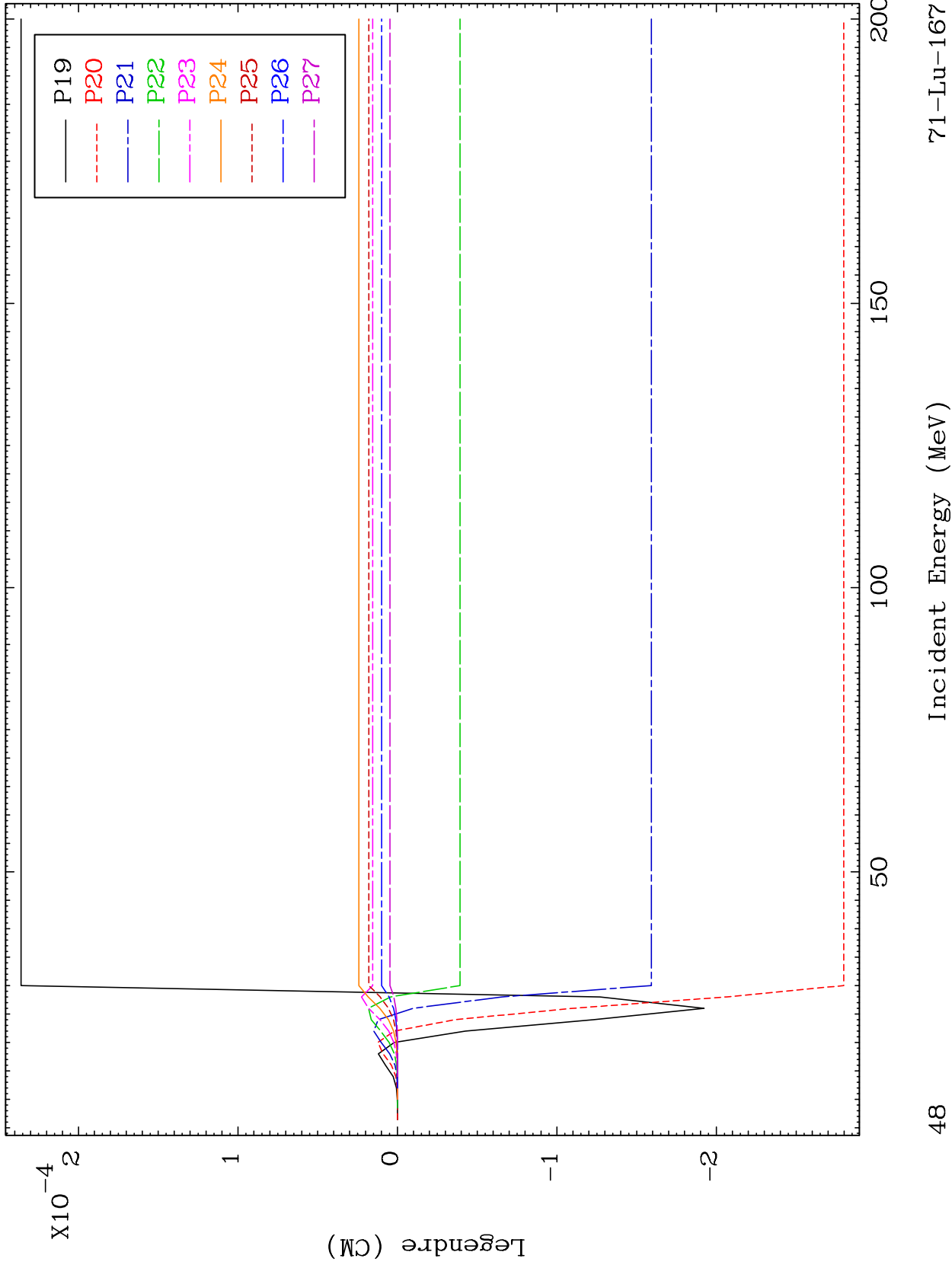
71-Lu-167



47

Incident Energy (MeV)

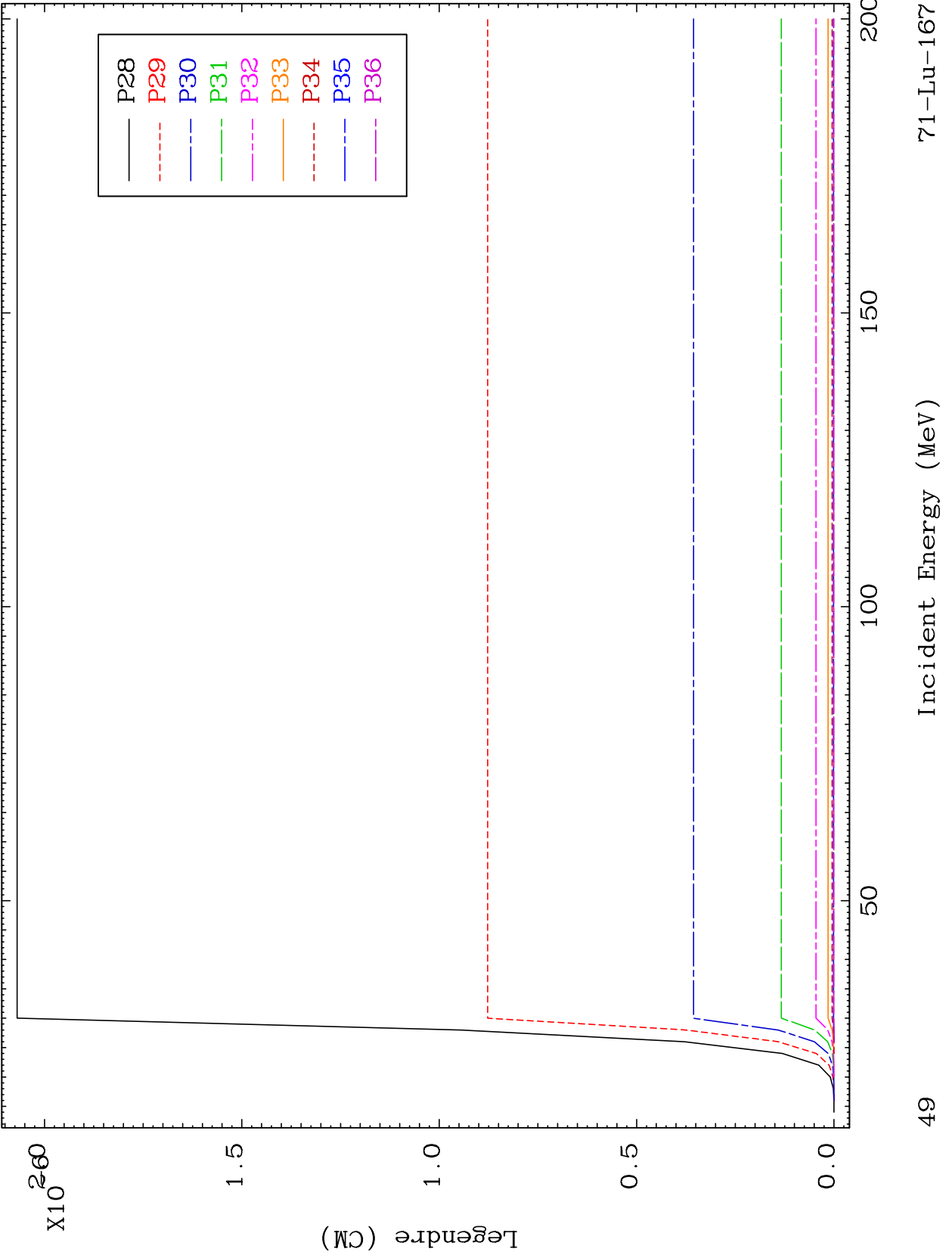
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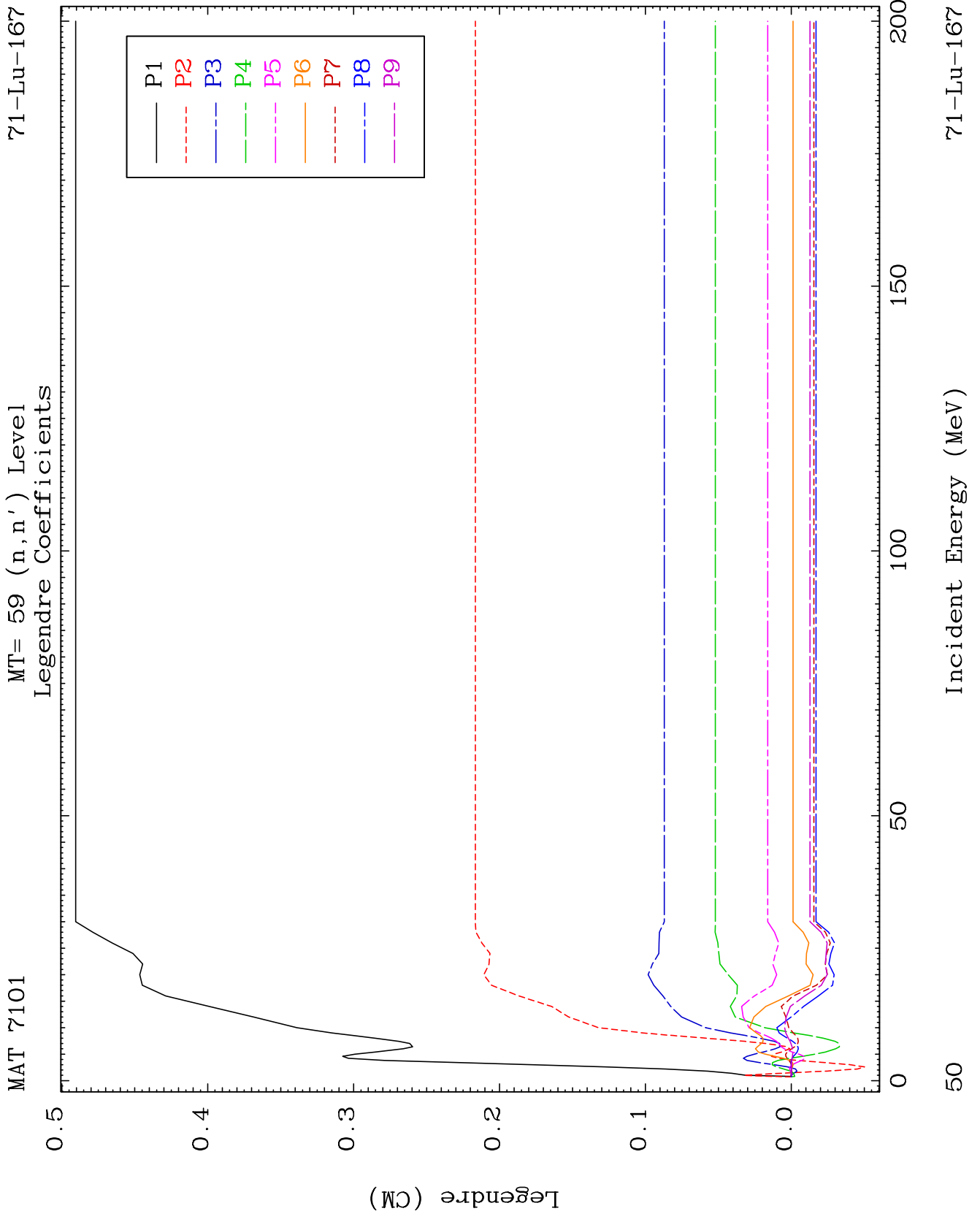


MAT 7101

MT= 58 (n,n') Level
Legendre Coefficients

71-Lu-167

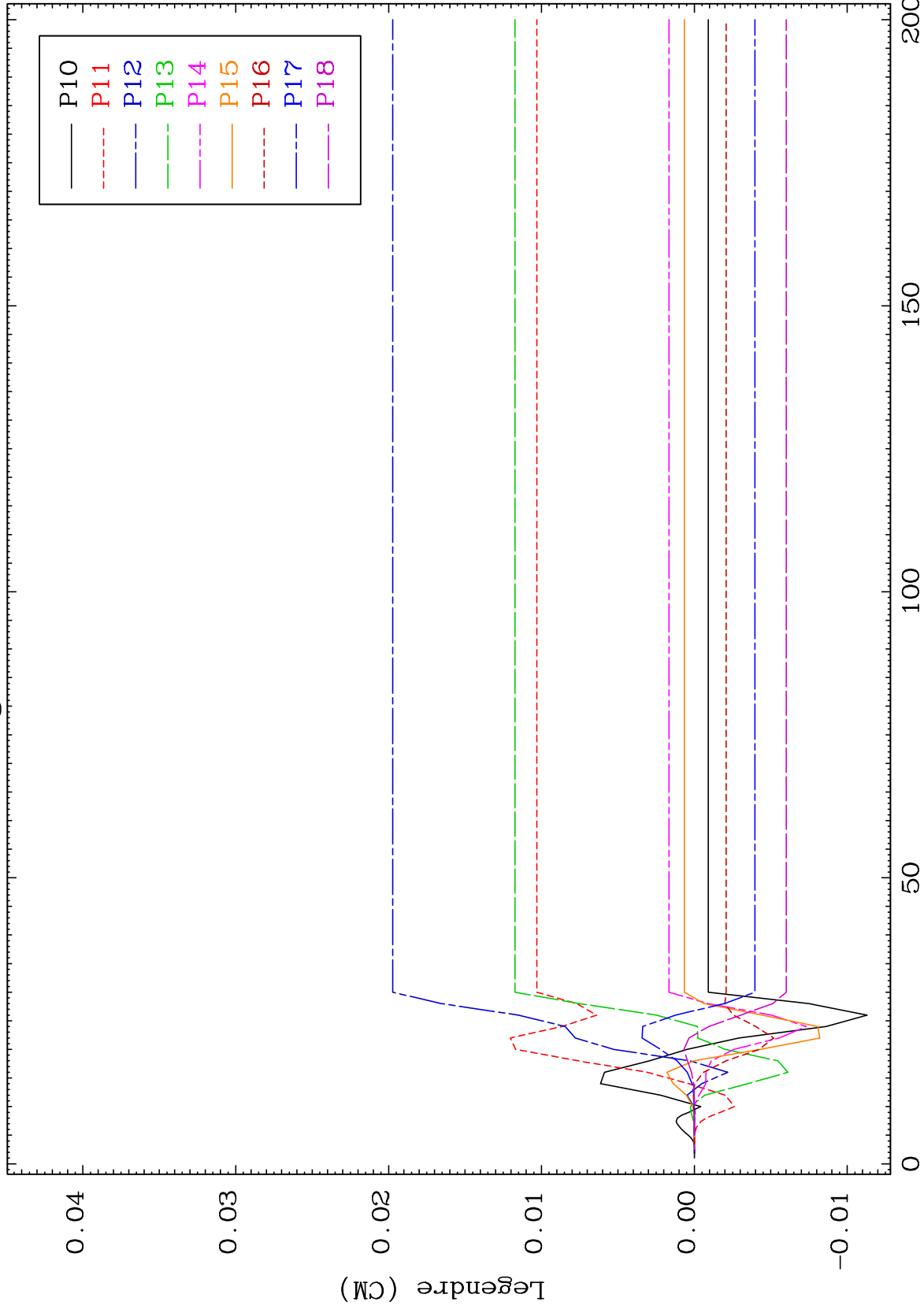




MAT 7101

MT= 59 (n,n') Level
Legendre Coefficients

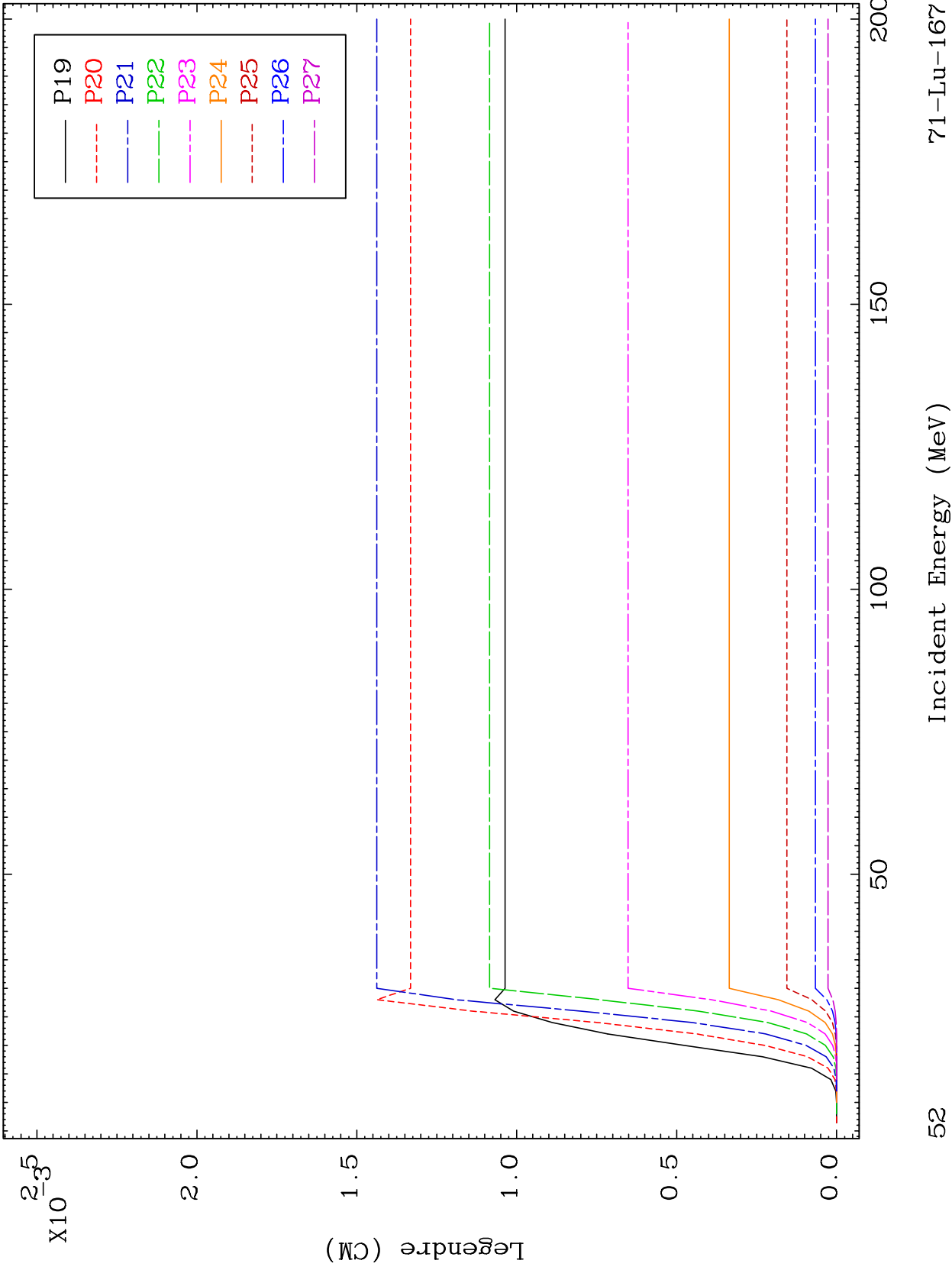
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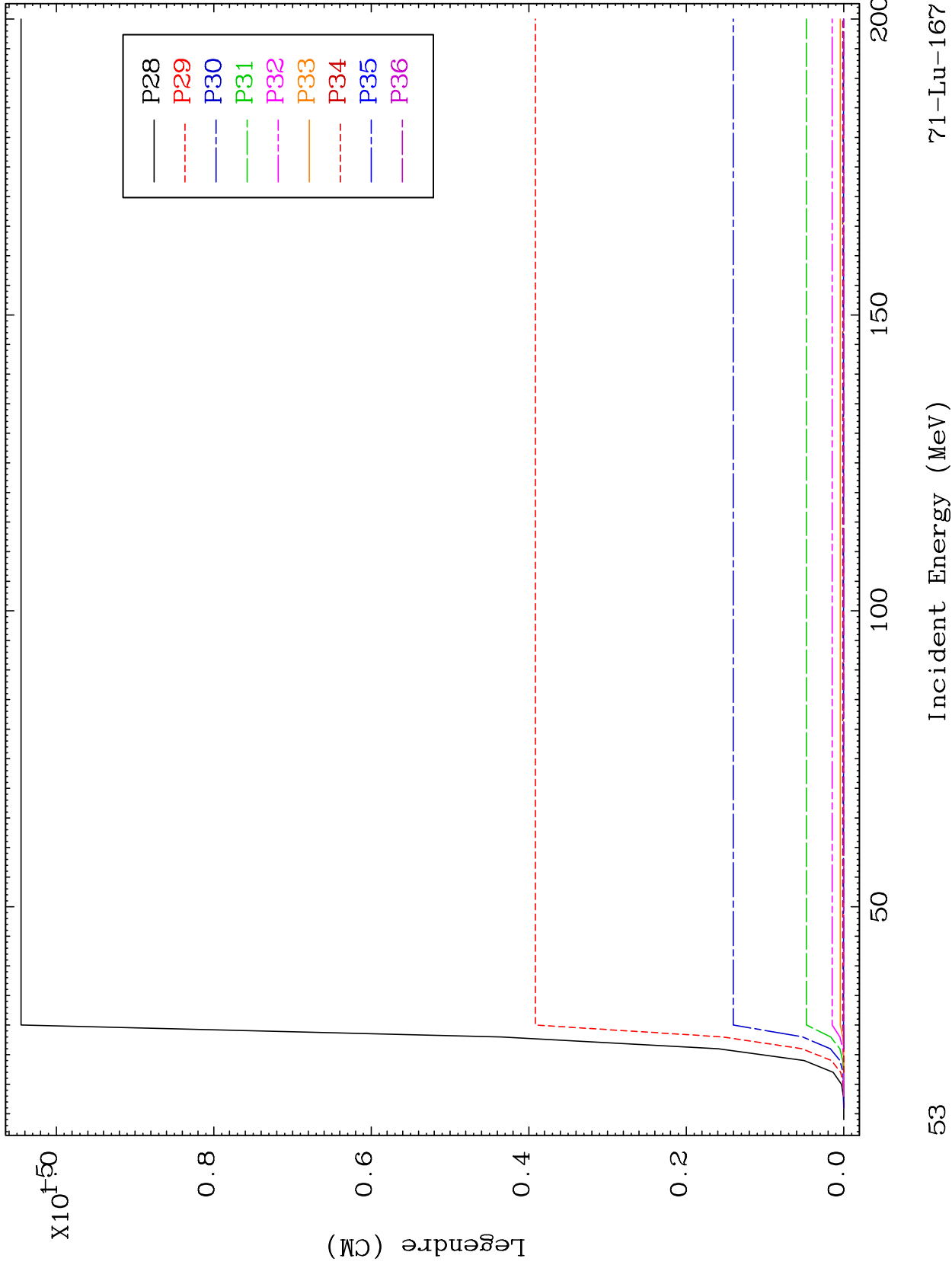


51

Incident Energy (MeV)

71-Lu-167

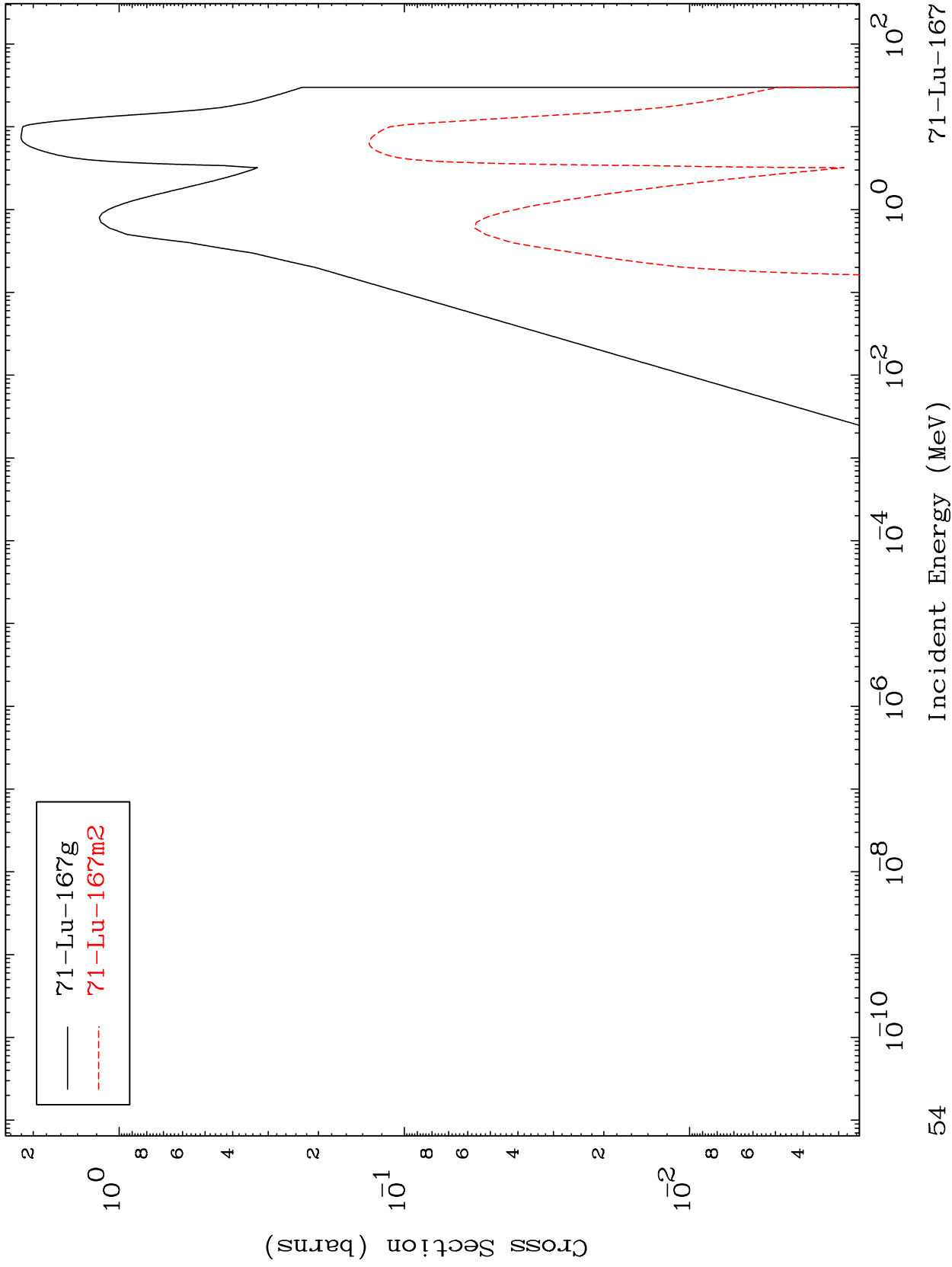




MAT 7101

Inelastic
Radionuclide Production Cross Section

⁷¹Lu-167

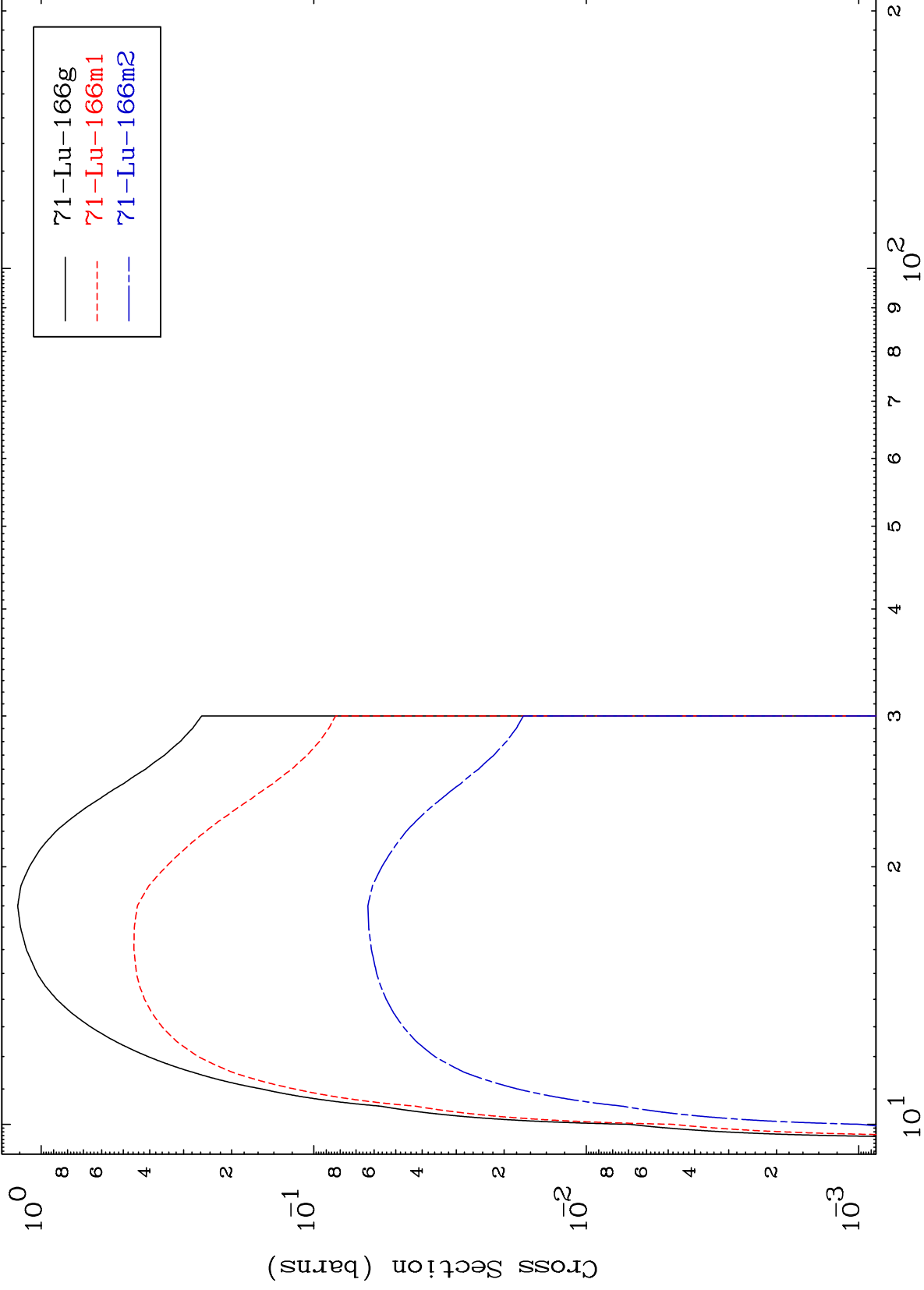


MAT 7101

(n,2n)

71-Lu-167

Radionuclide Production Cross Section



Incident Energy (MeV)

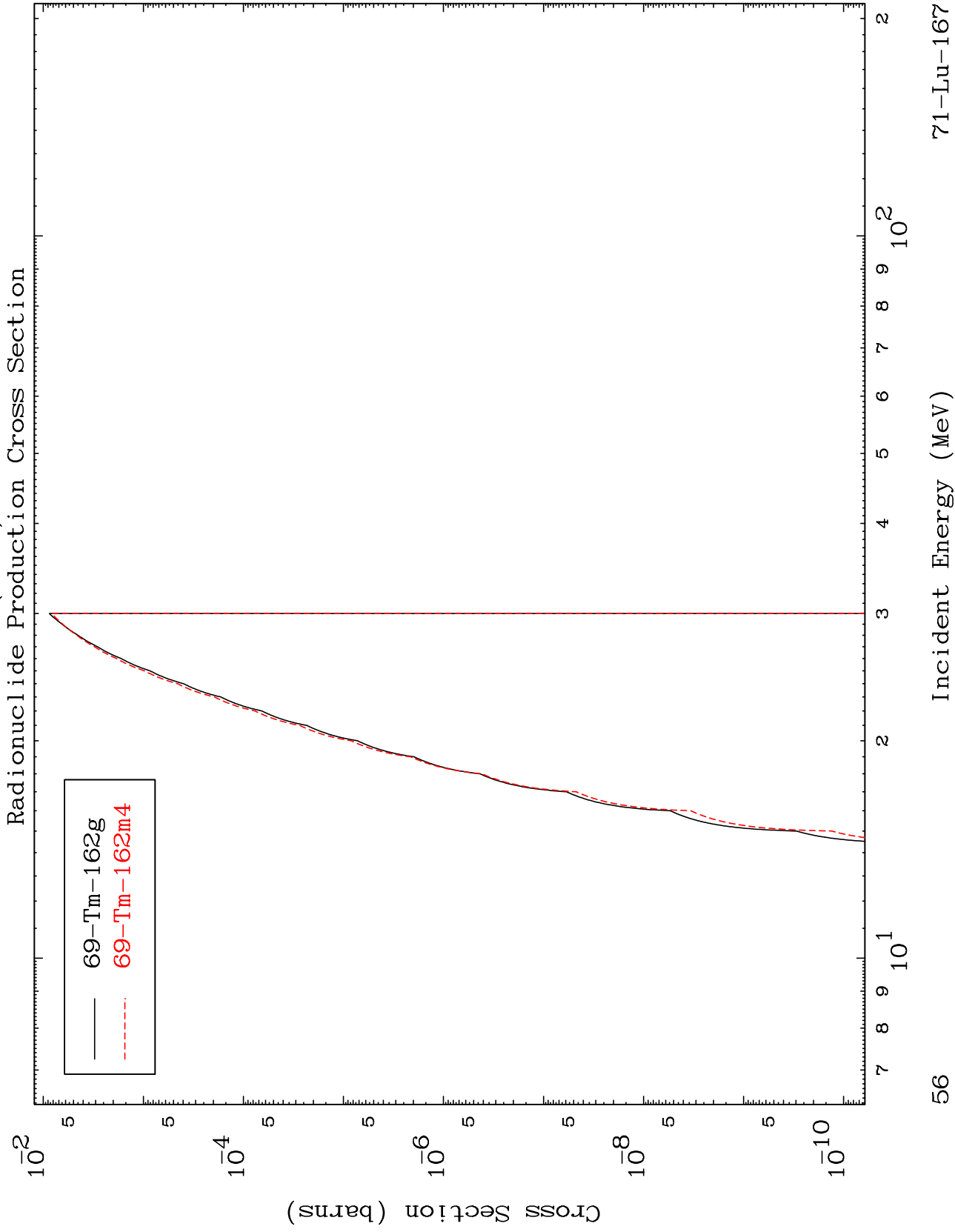
71-Lu-167

55

MAT 7101

$(n,2n) \alpha$

71-Lu-167



56

Incident Energy (MeV)

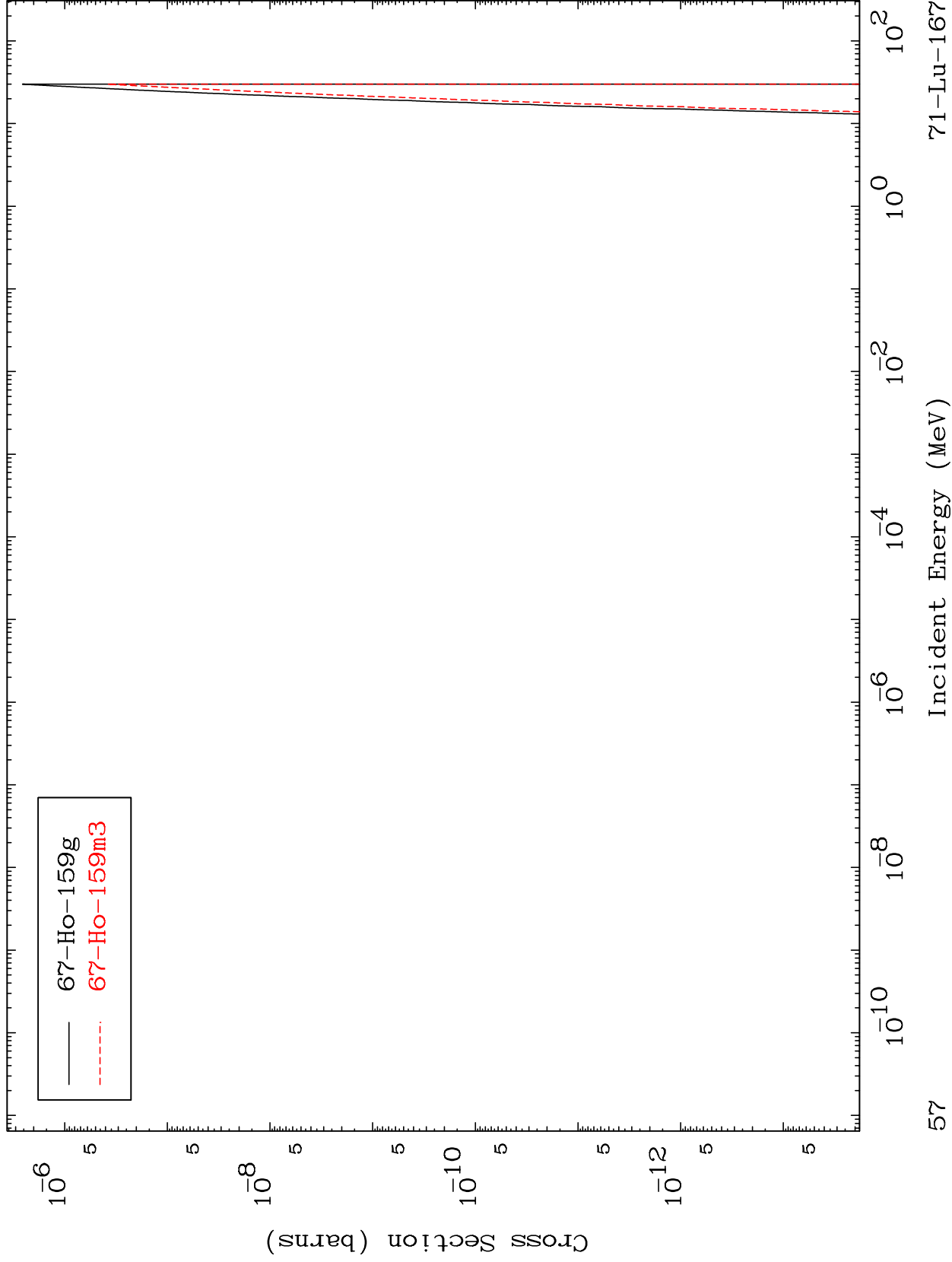
71-Lu-167

MAT 7101

(n,n') 2α

71-Lu-167

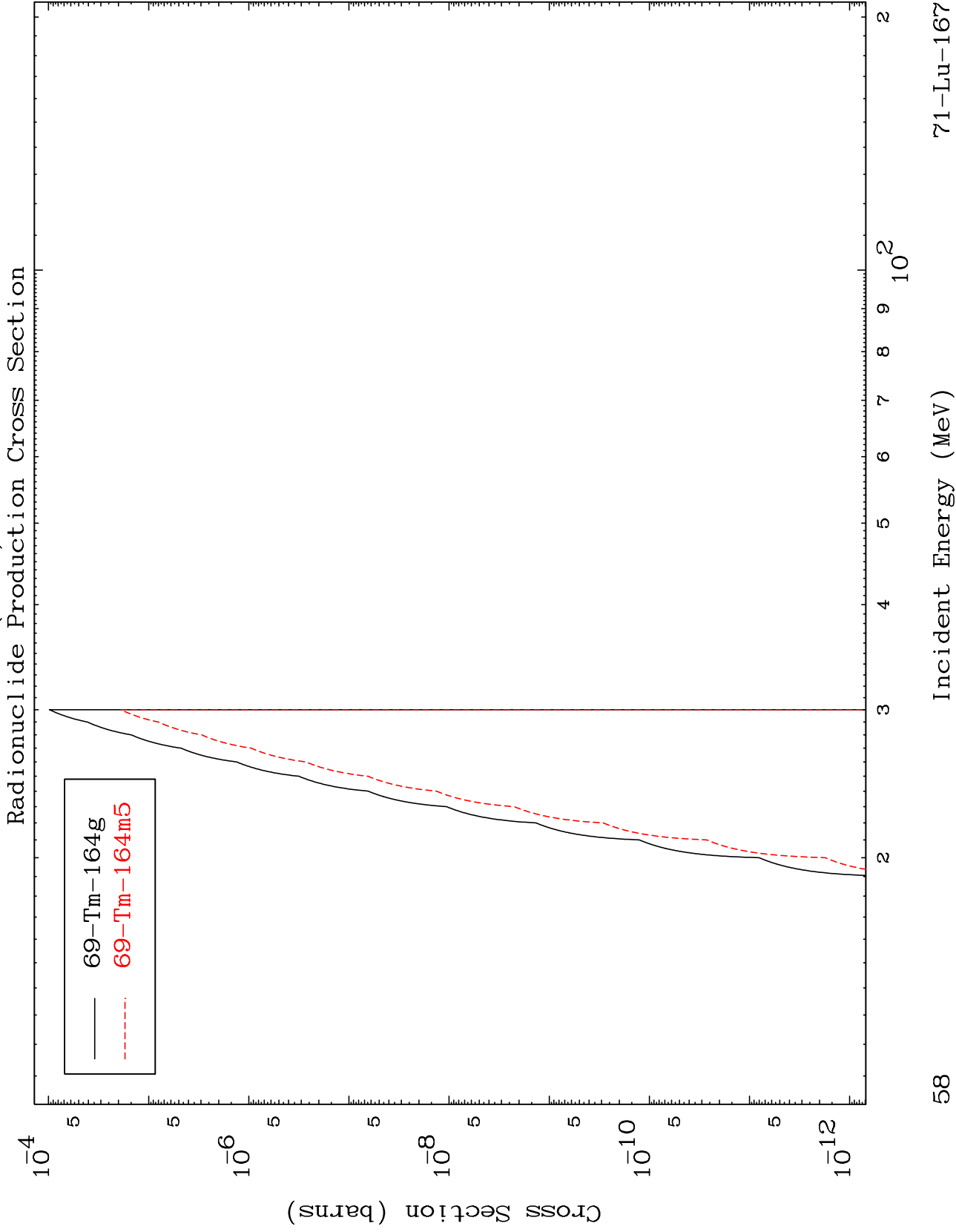
Radionuclide Production Cross Section



MAT 7101

(n,n') He-3

71-Lu-167



58

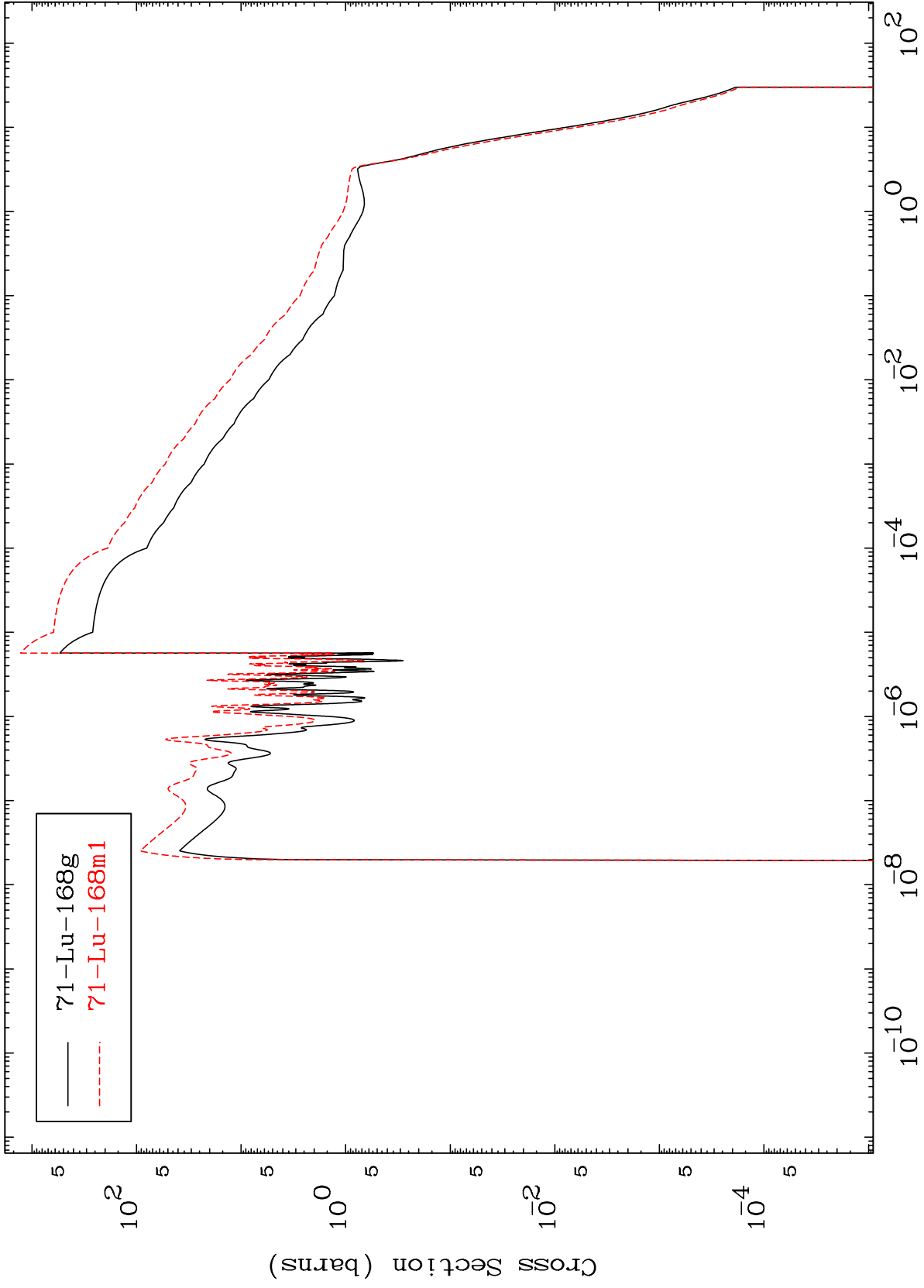
Incident Energy (MeV)

71-Lu-167

MAT 7101

71-Lu-167

Radionuclide Production Cross Section



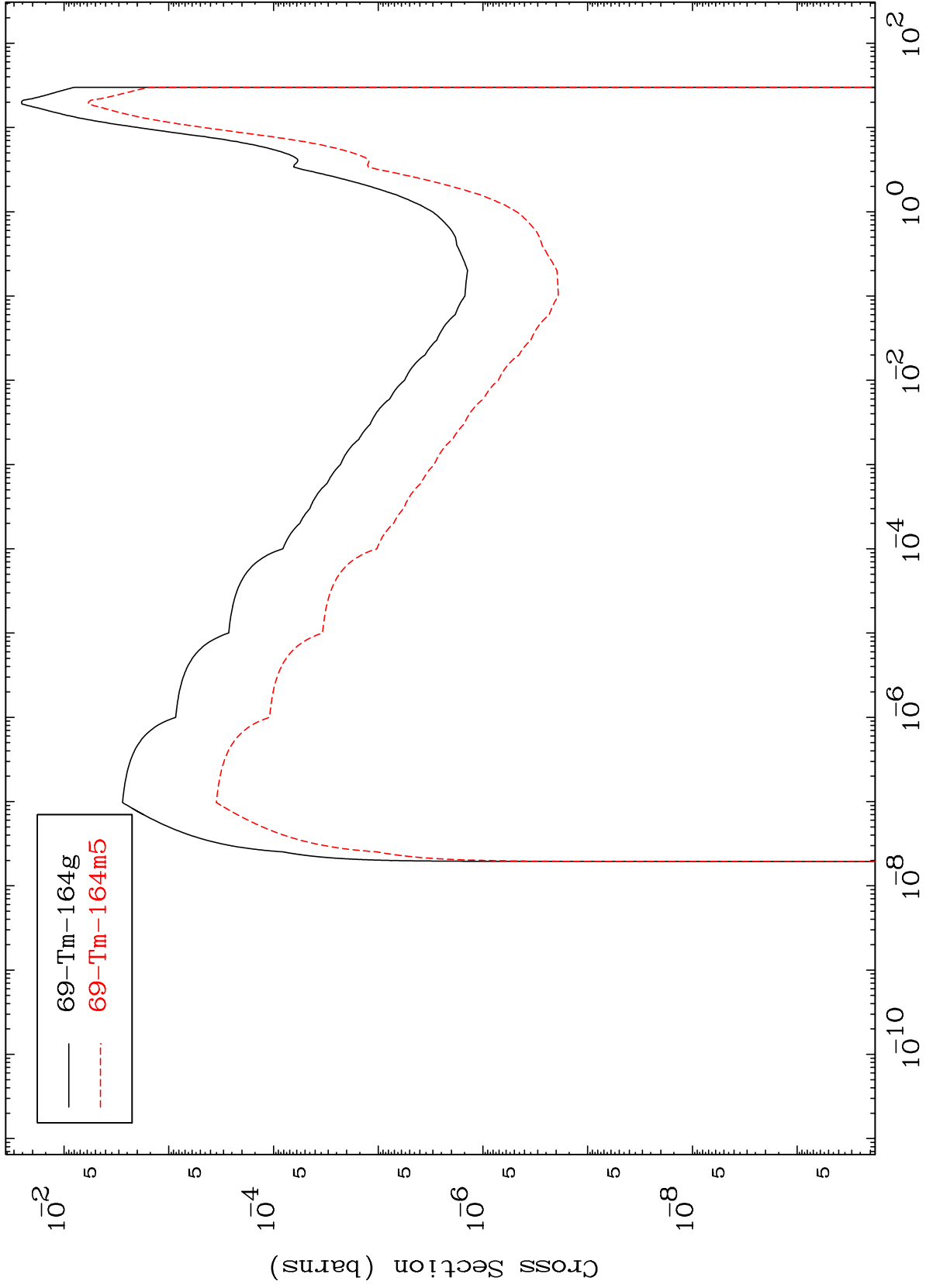
59

71-Lu-167

MAT 7101

71-Lu-167

(n, α)
Radionuclide Production Cross Section



60

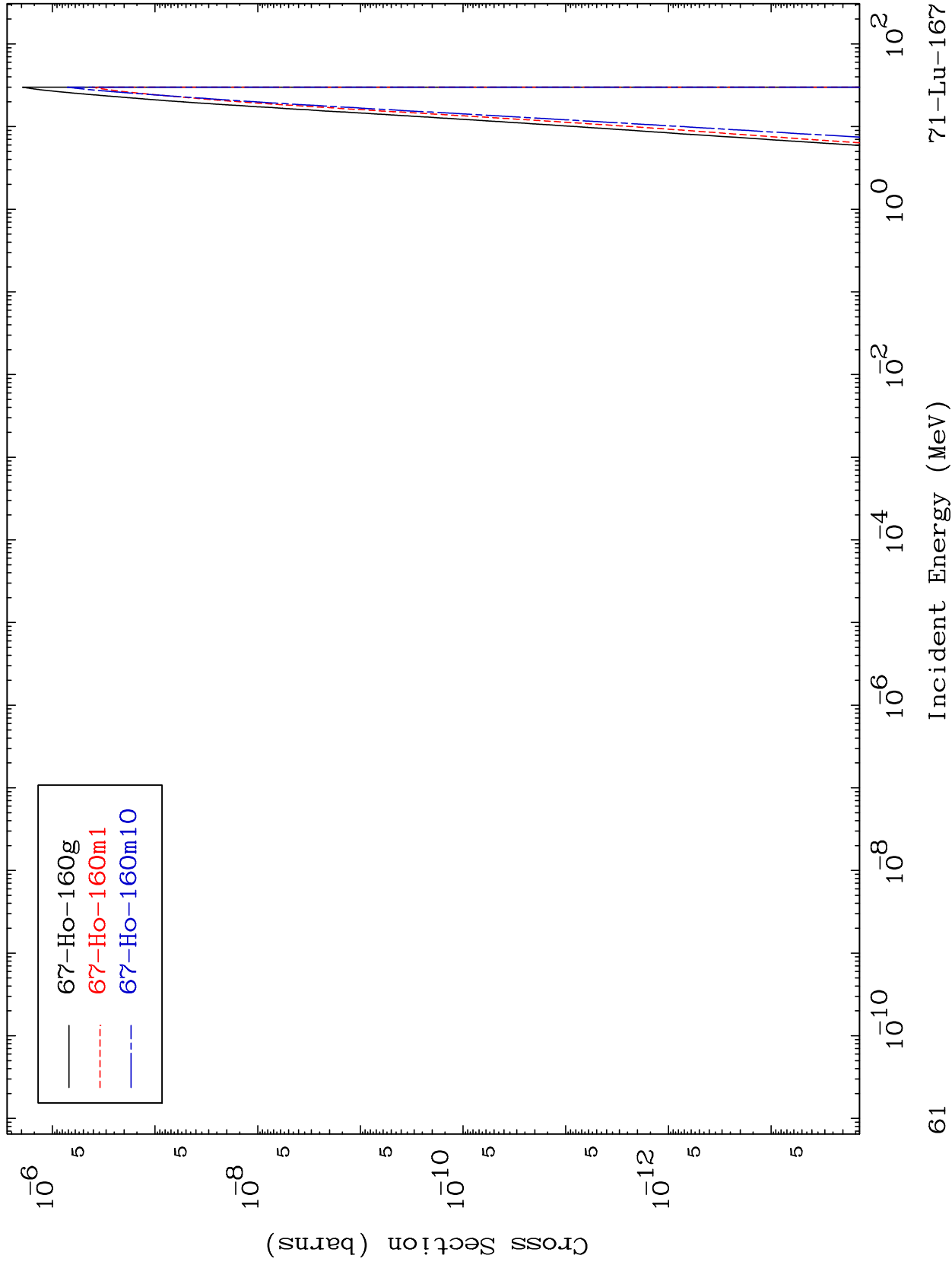
Incident Energy (MeV)

71-Lu-167

MAT 7101

⁷¹Lu-167

(n,2α)
Radionuclide Production Cross Section



61

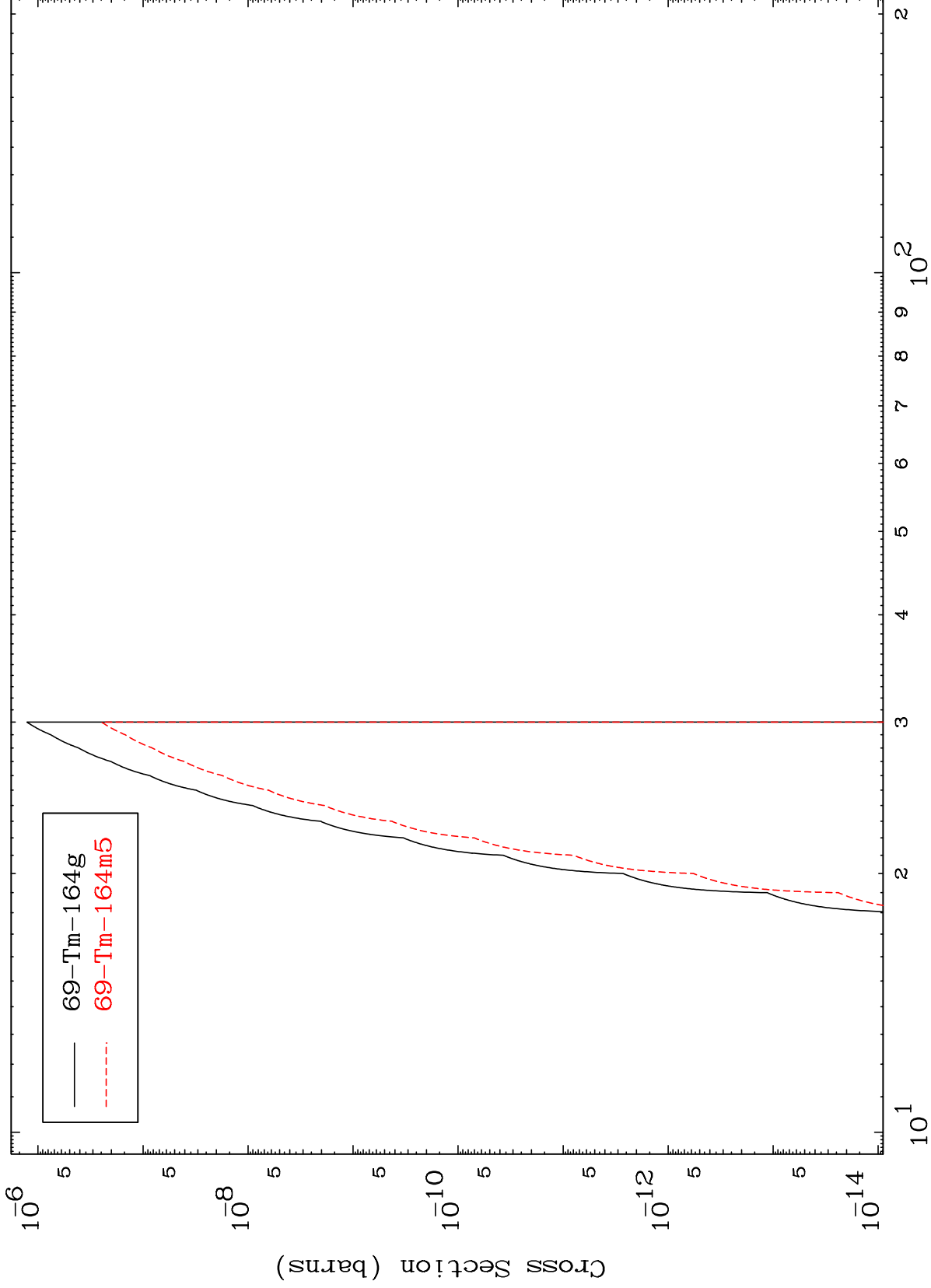
⁷¹Lu-167

MAT 7101

(n,p) t

⁷¹Lu-167

Radionuclide Production Cross Section



Incident Energy (MeV)

⁷¹Lu-167

62