

Program EVALPLOT
(Version 2021-1)

by

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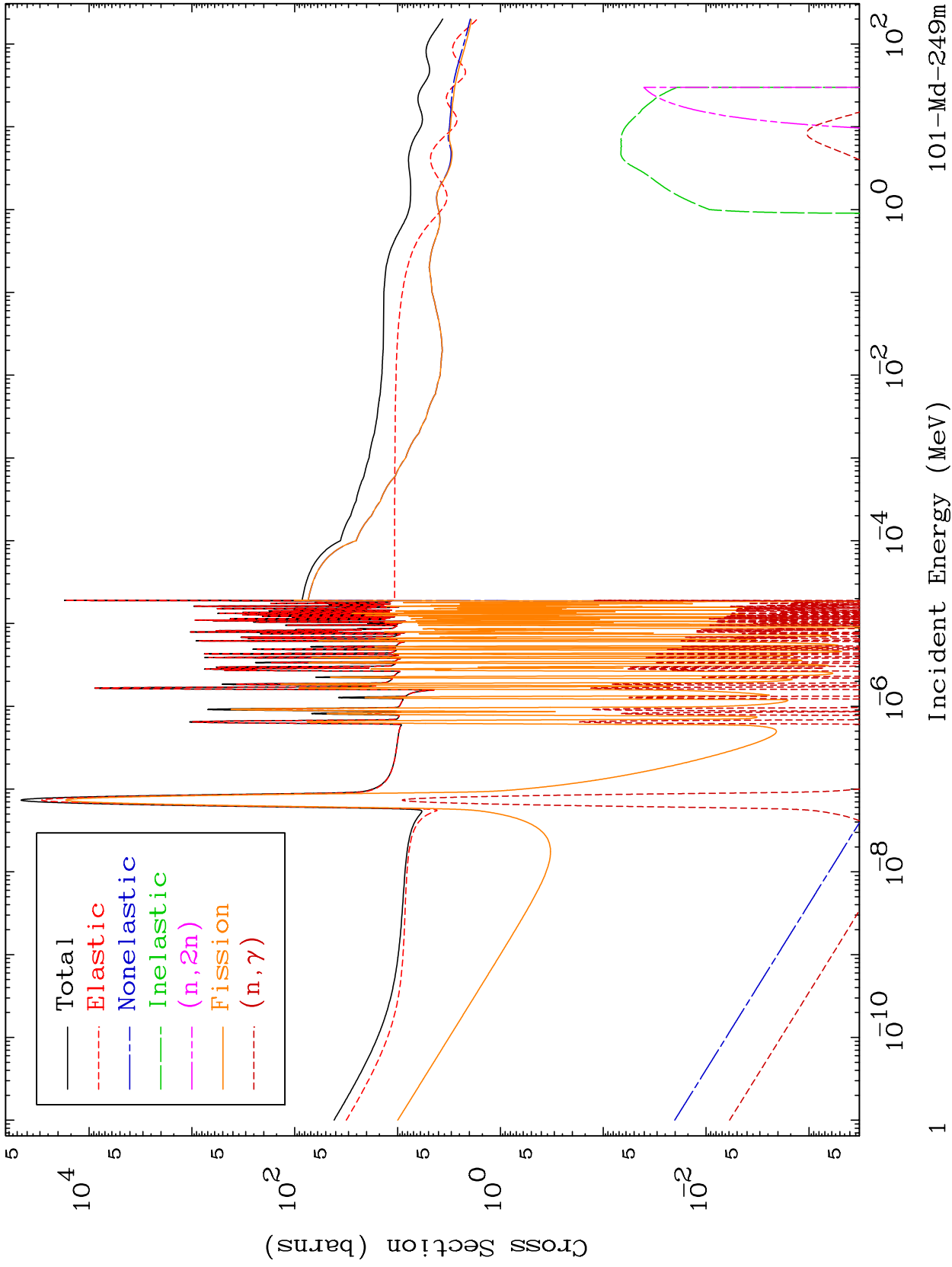
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Press Mouse Button to Start

MAT 9955

Neutron Major
293 Kelvin Cross Sections

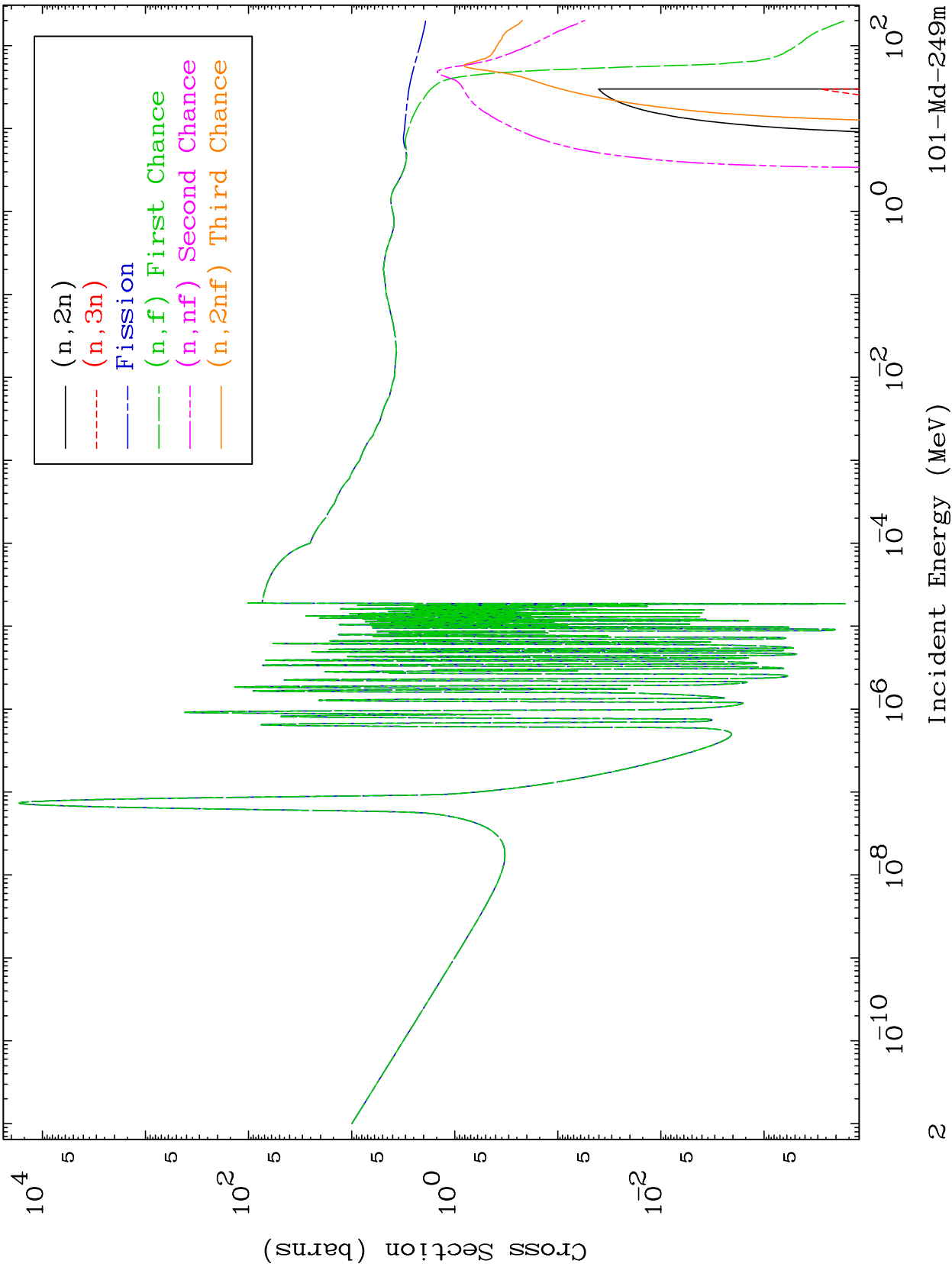
101-Md-249m



MAT 9955

Neutron Absorption
293 Kelvin Cross Sections

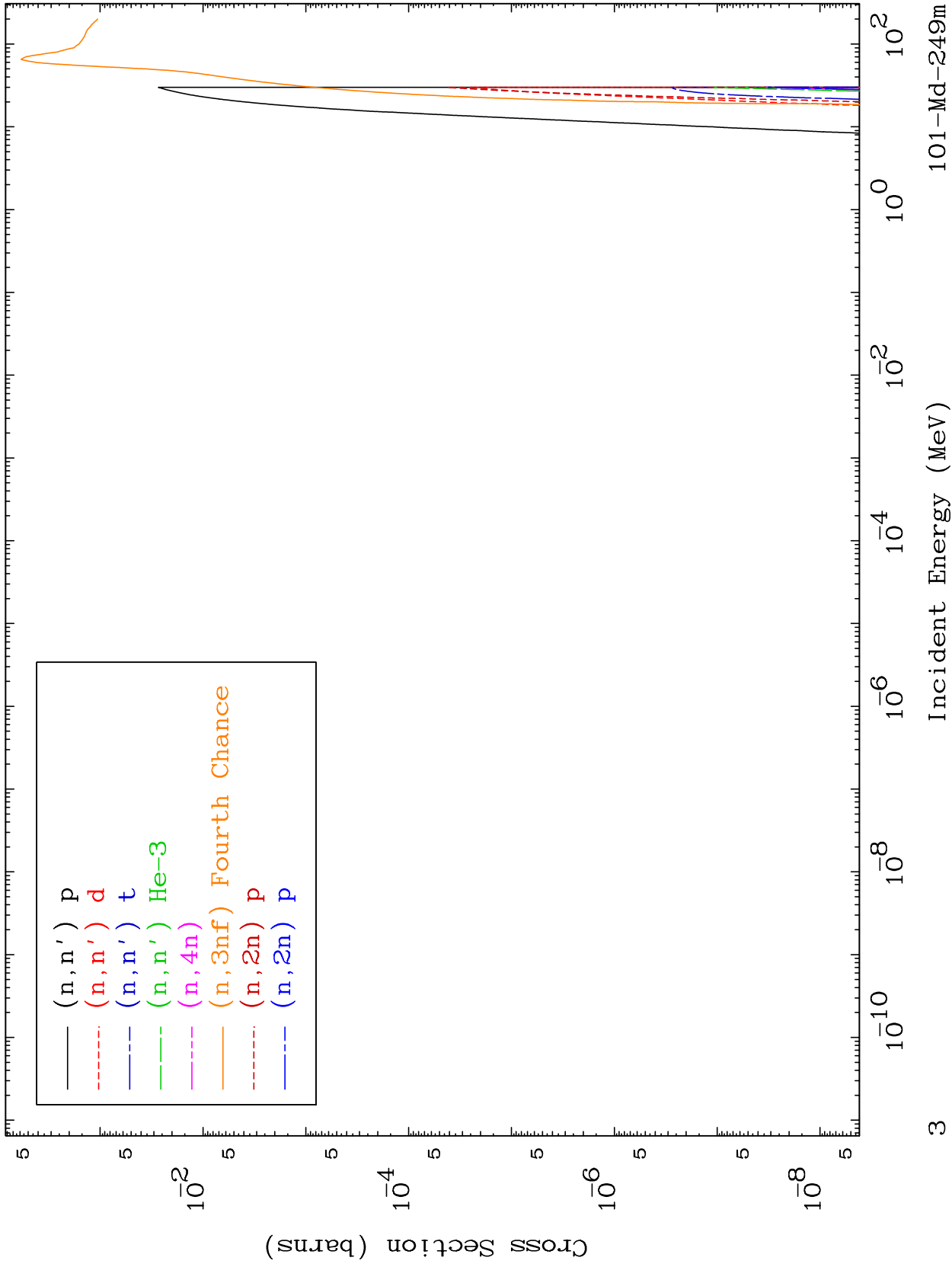
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Neutron Absorption
293 Kelvin Cross Sections

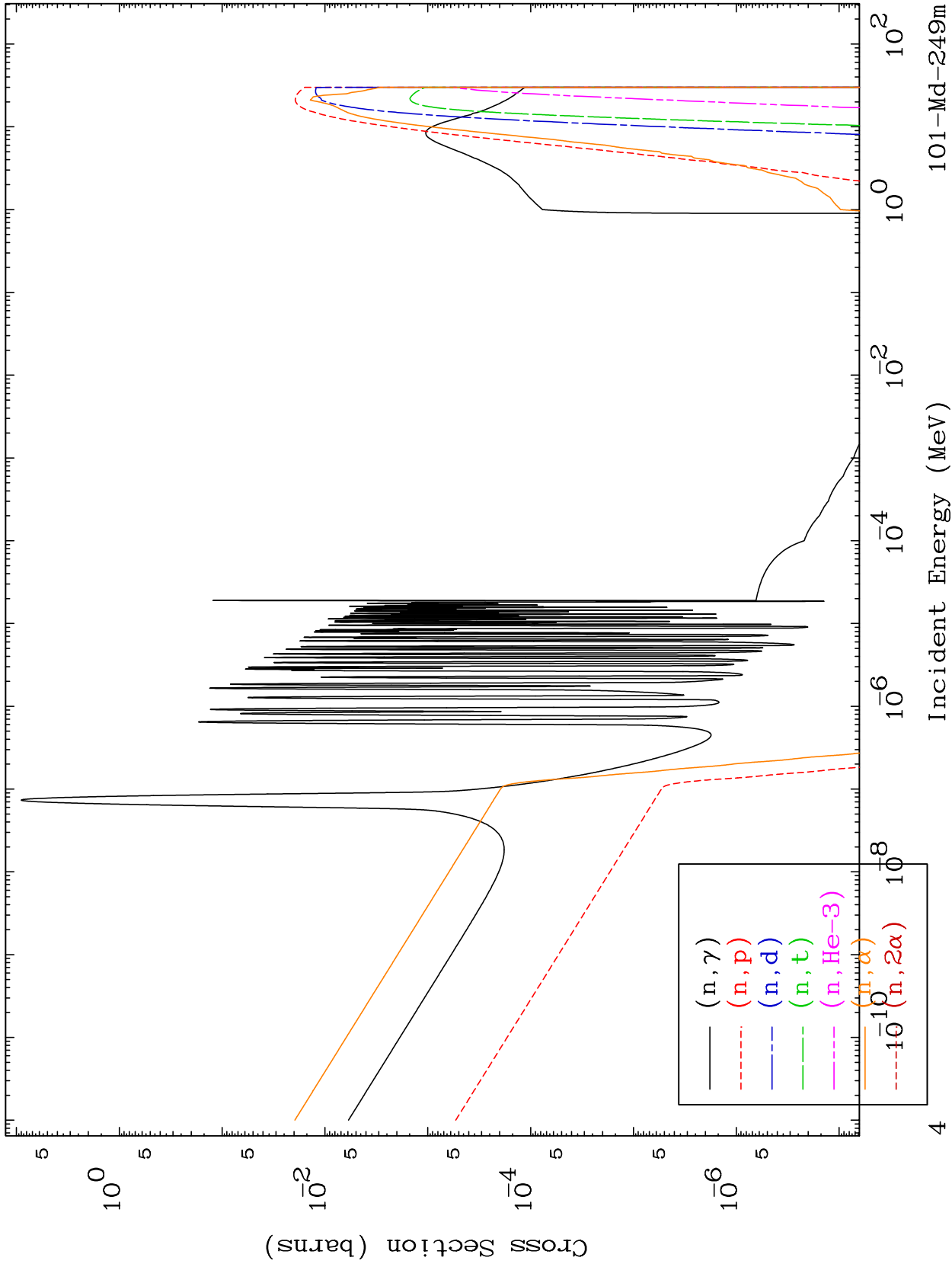
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Neutron Absorption
293 Kelvin Cross Sections

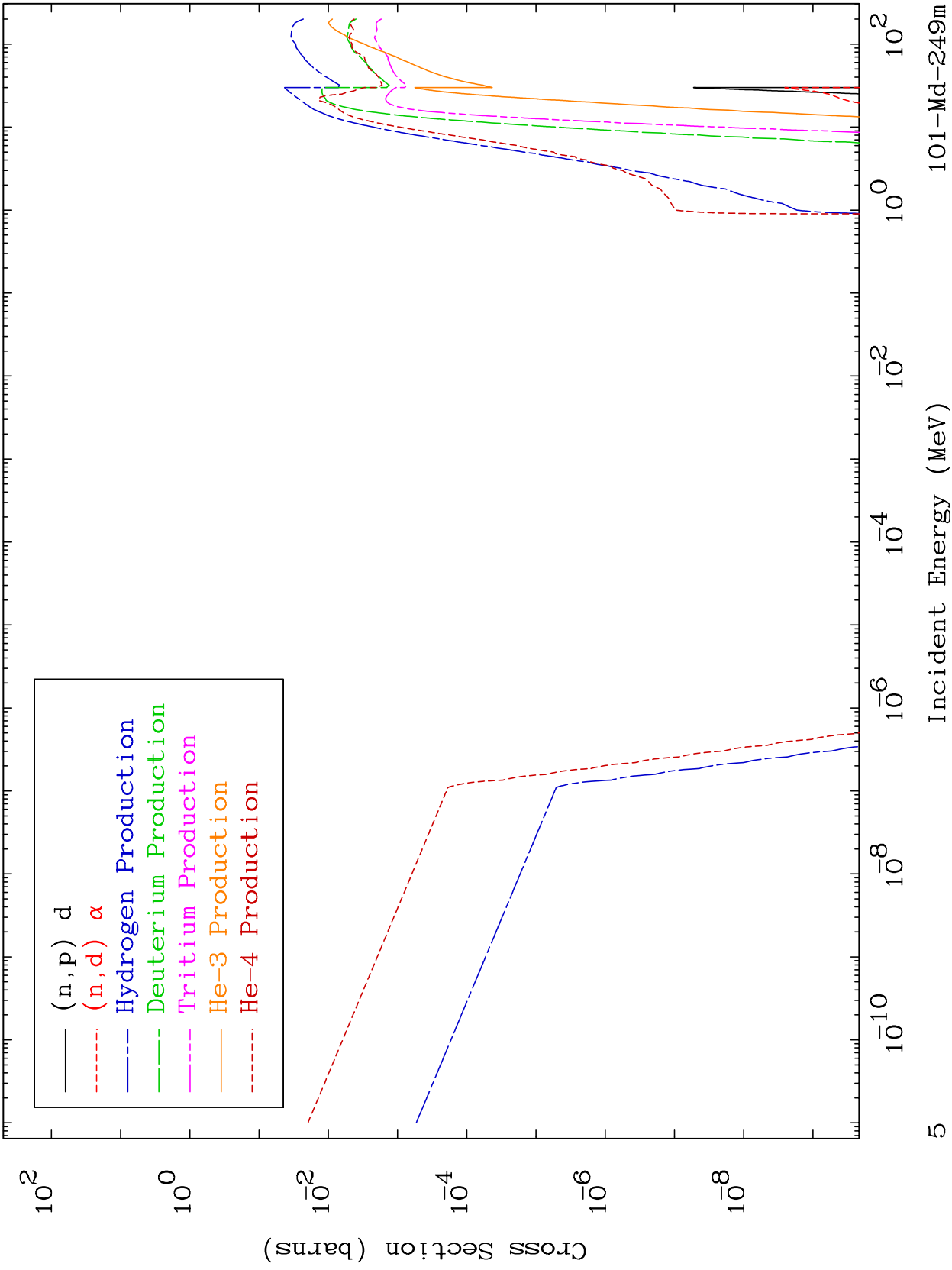
101-Md-249m



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Neutron Absorption
293 Kelvin Cross Sections

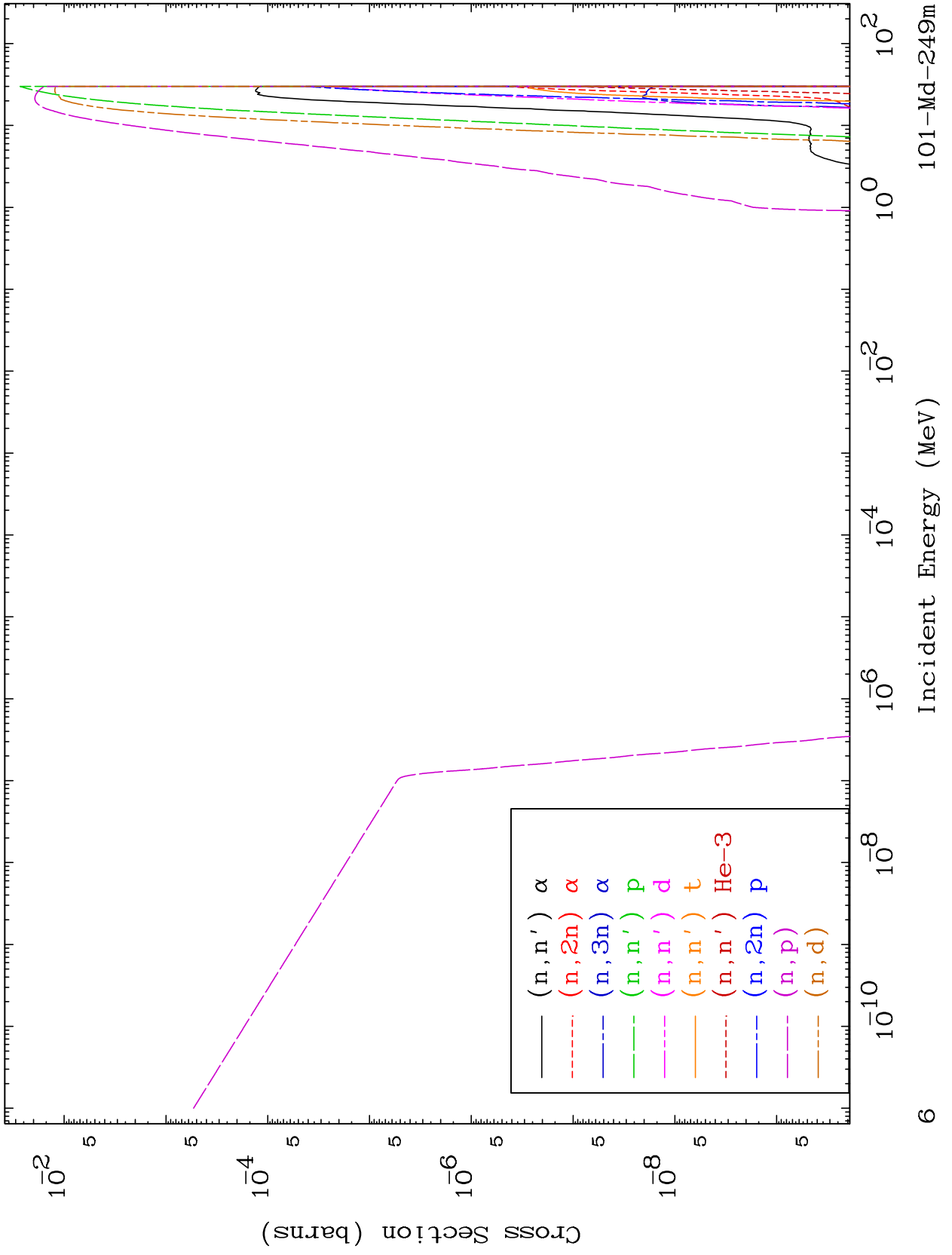
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Charged Particle
293 Kelvin Cross Sections

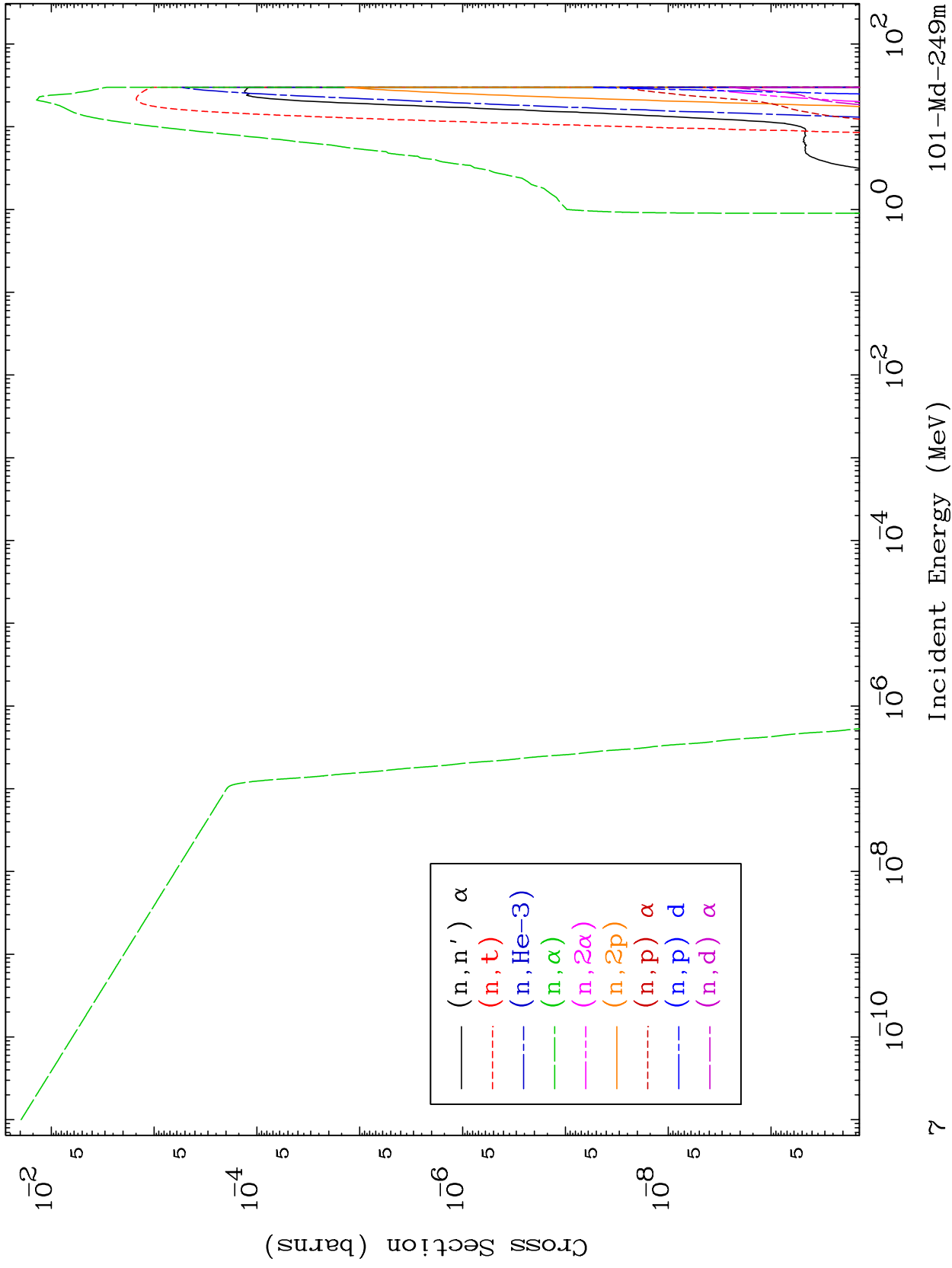
101-Md-249m



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Charged Particle
293 Kelvin Cross Sections

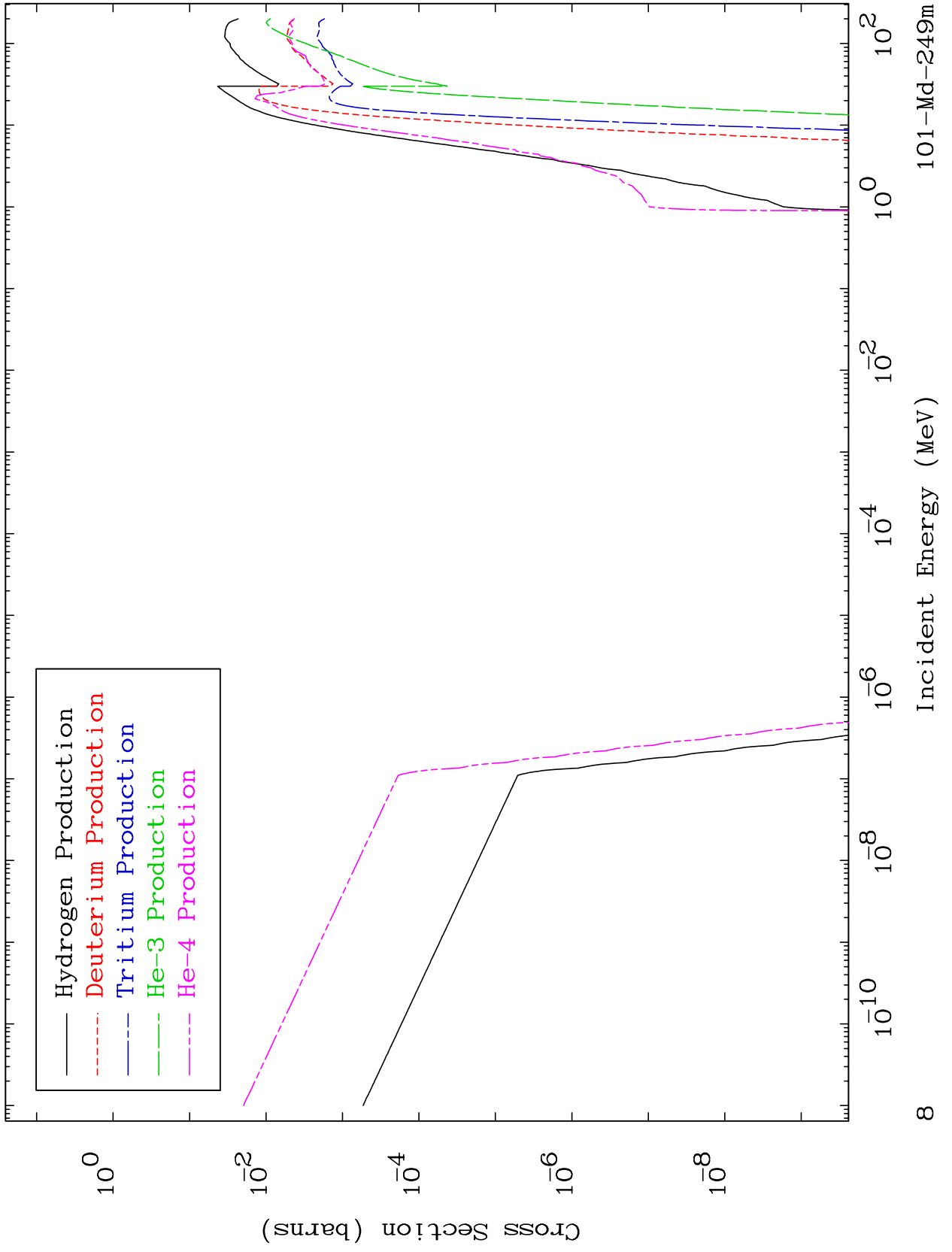
101-Md-249m



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Particle Production
293 Kelvin Cross Sections

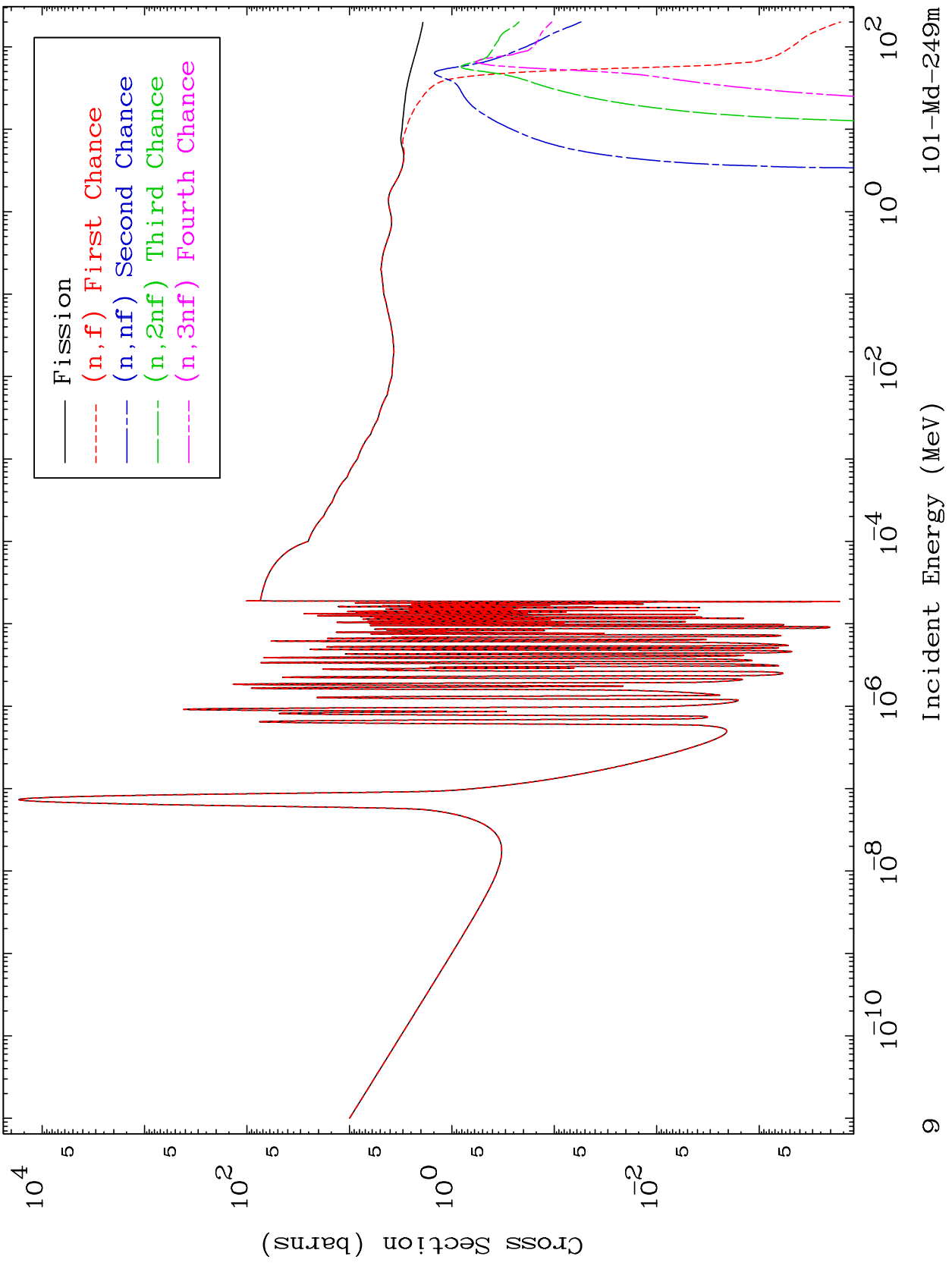
101-Md-249m



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Fission
293 Kelvin Cross Sections

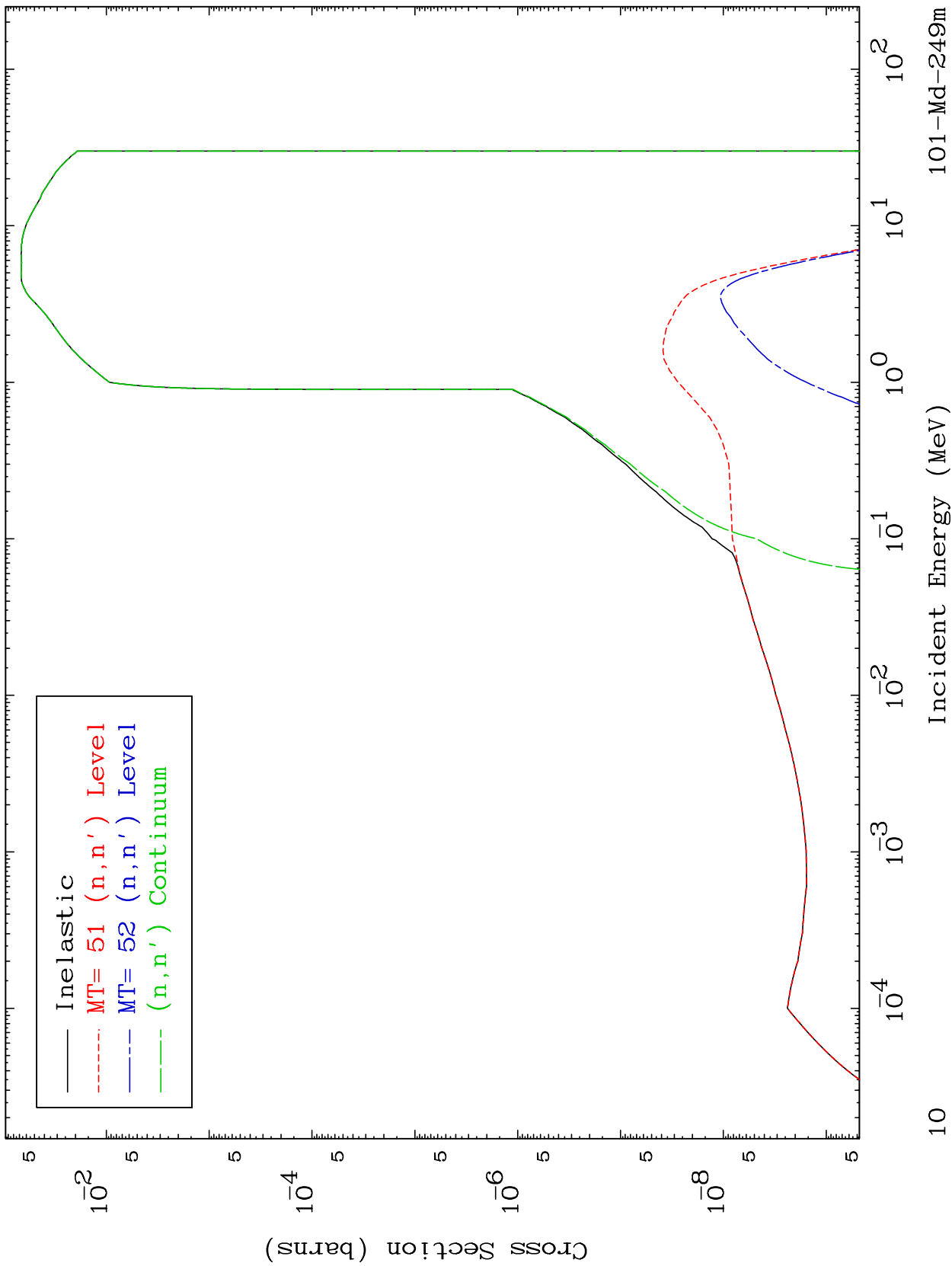
101-Md-249m



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293 Kelvin Cross Sections
(n,n') Levels

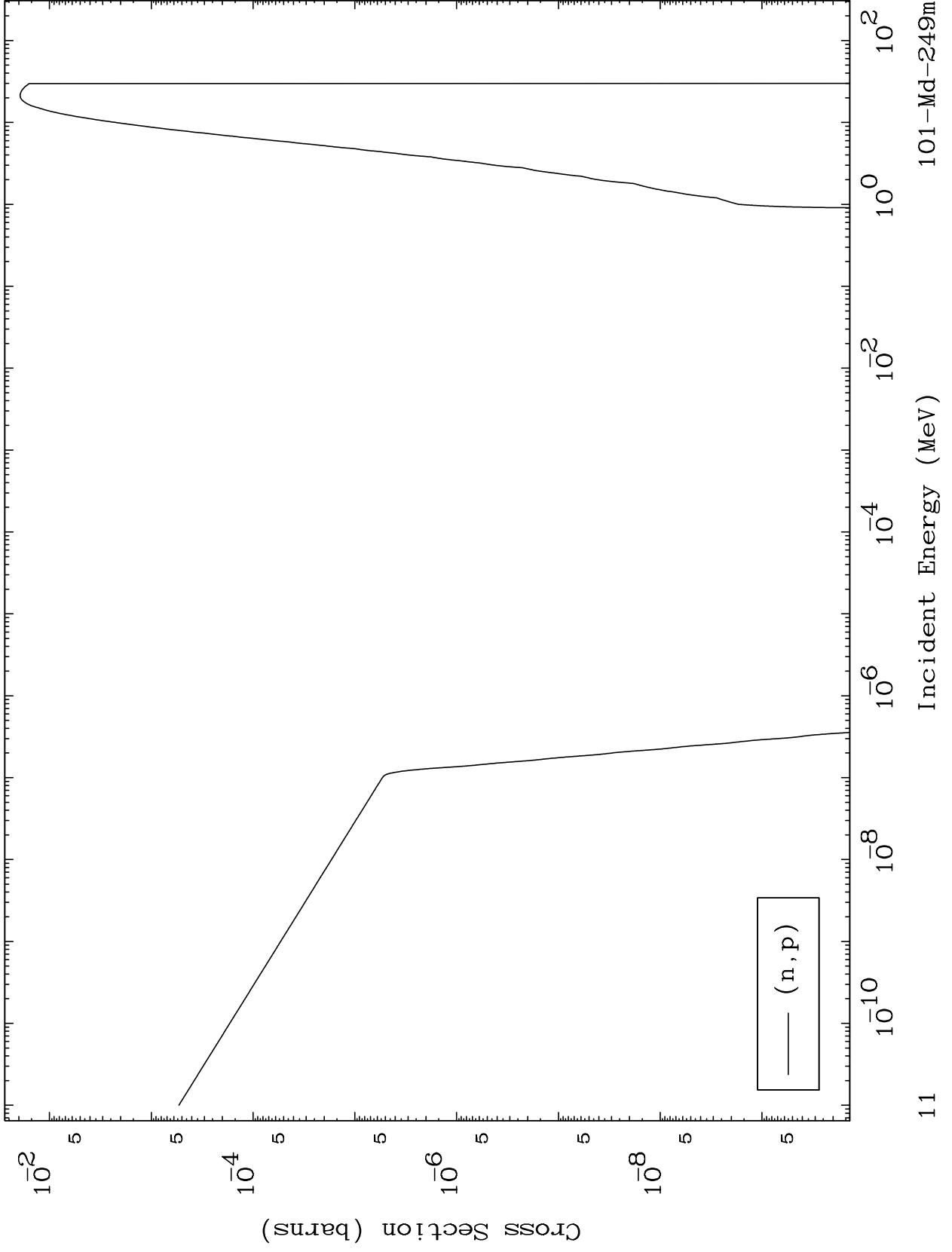
101-Md-249m



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(n,p) Levels
293 Kelvin Cross Sections

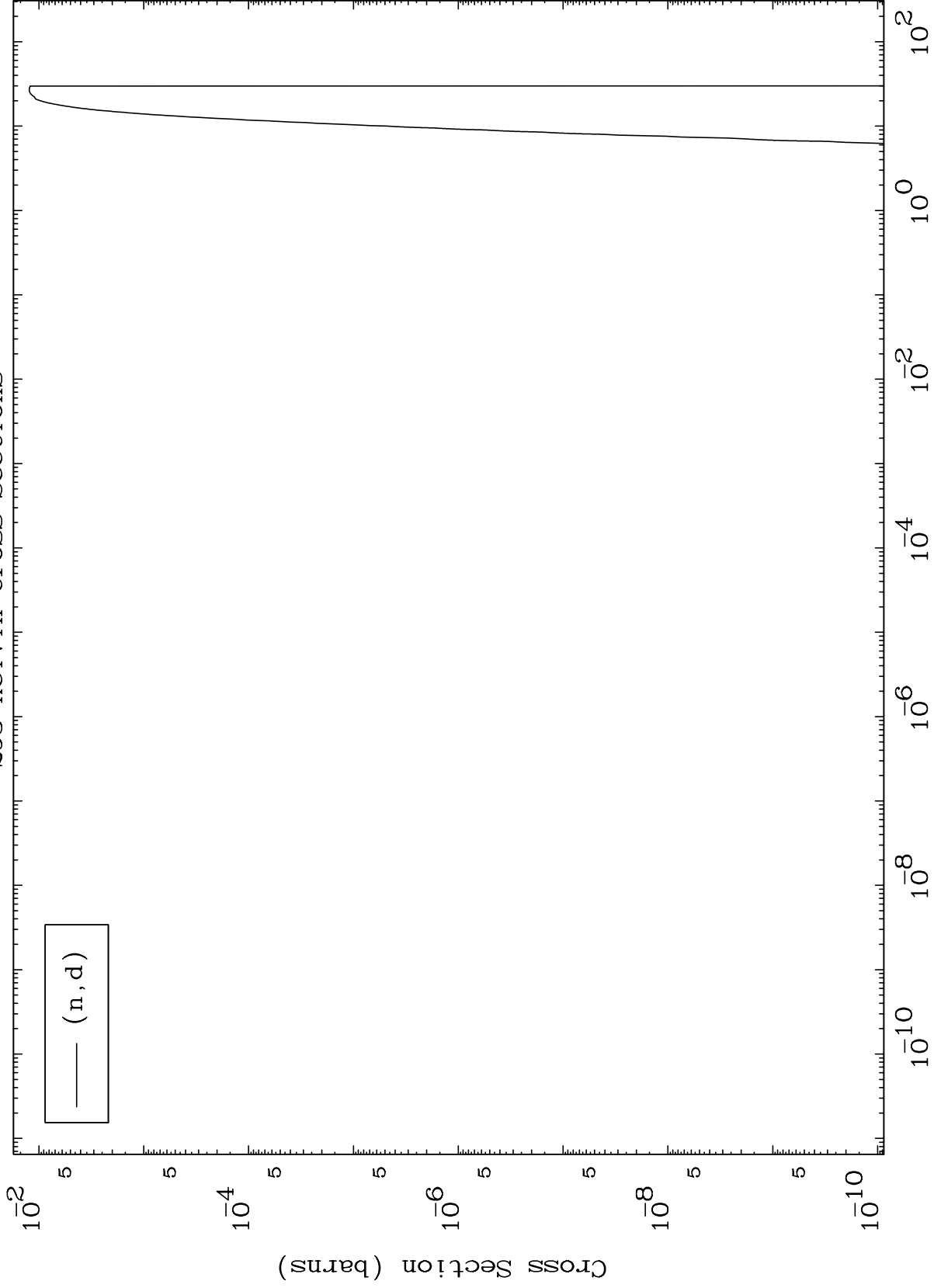
101-Md-249m



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(n,d) Levels
293 Kelvin Cross Sections

101-Md-249m



12

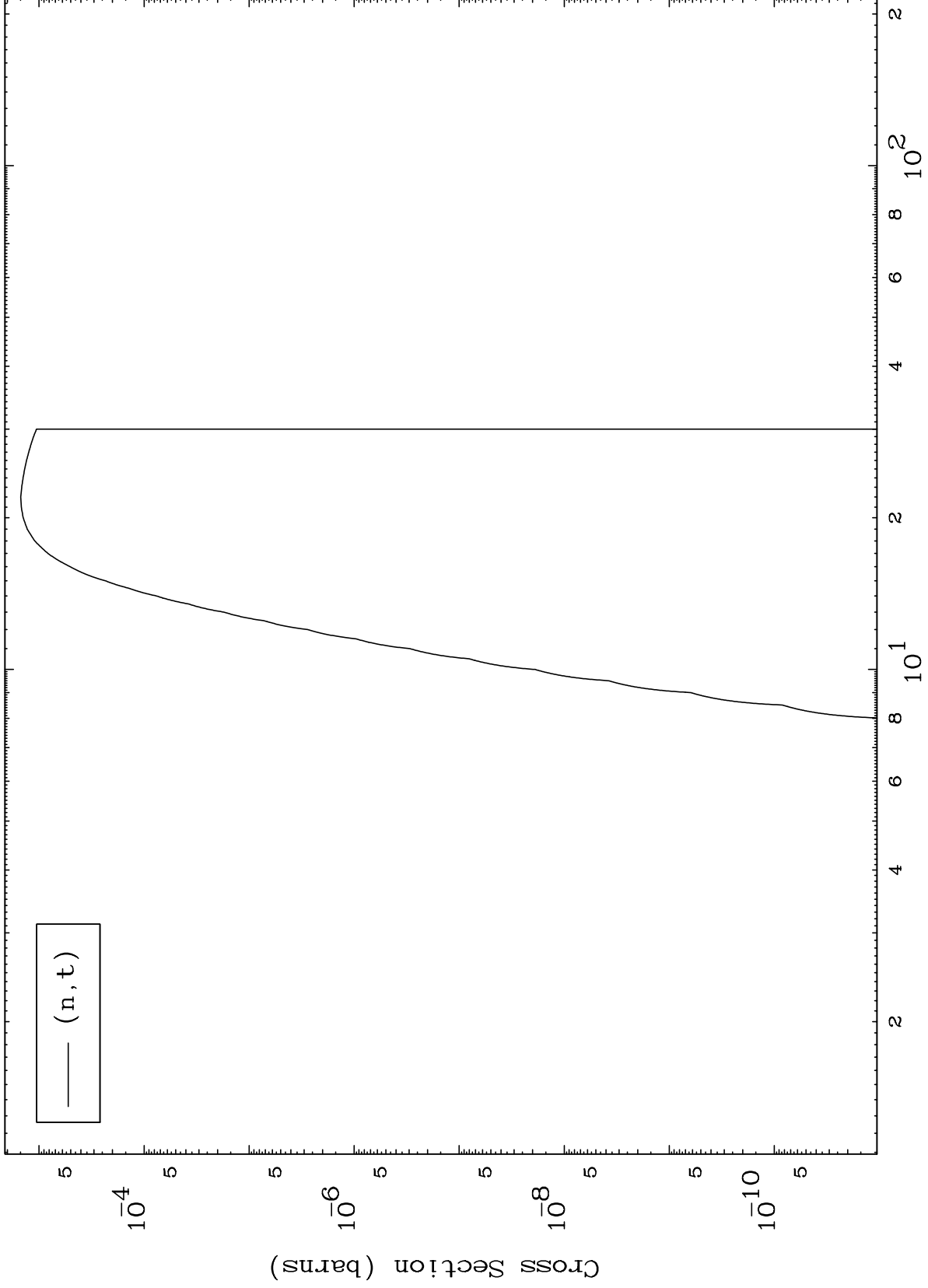
Incident Energy (MeV)

101-Md-249m

MAT 9955

(n,t) Levels
293 Kelvin Cross Sections

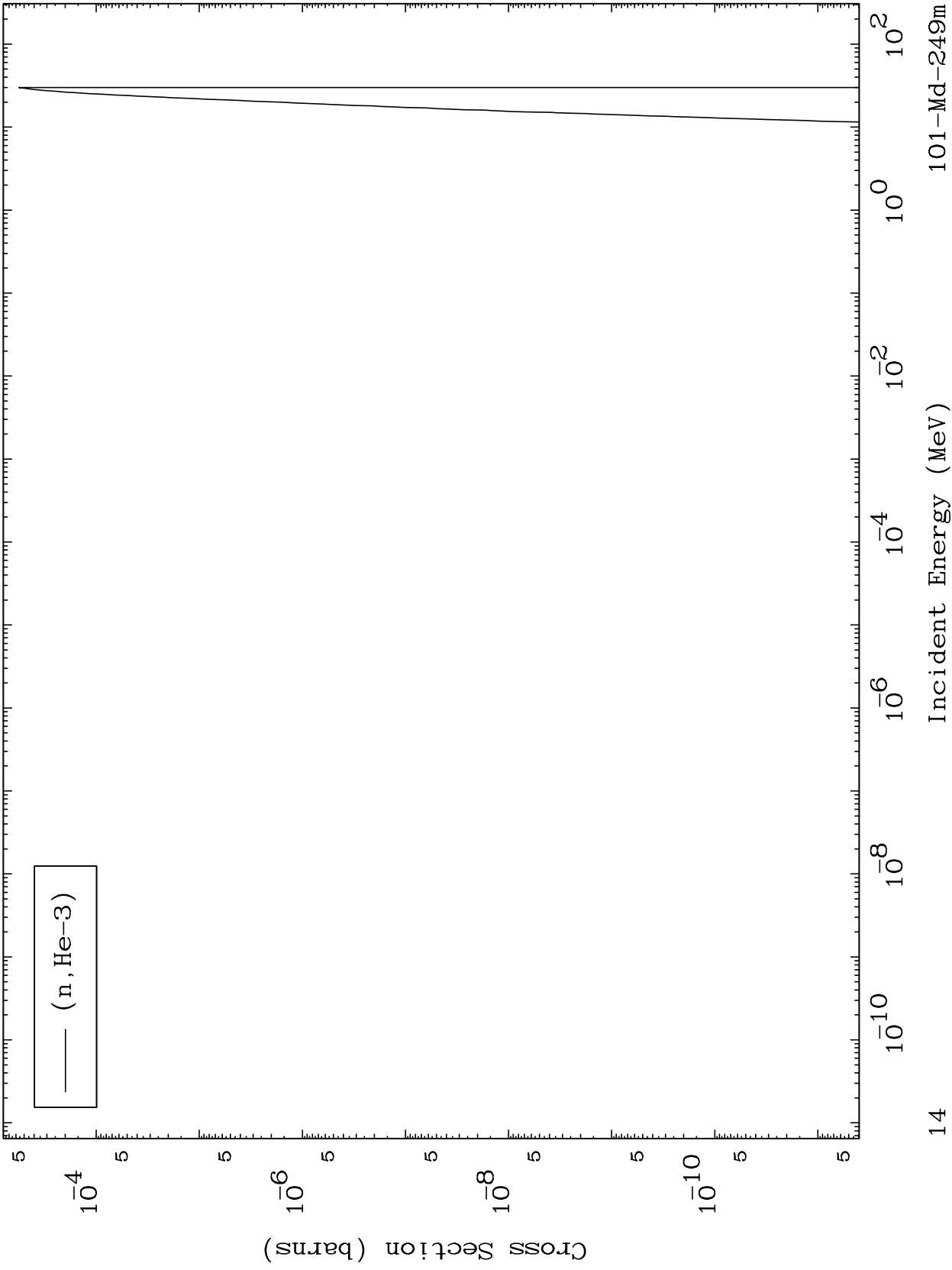
101-Md-249m



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(n,He3) Levels
293 Kelvin Cross Sections

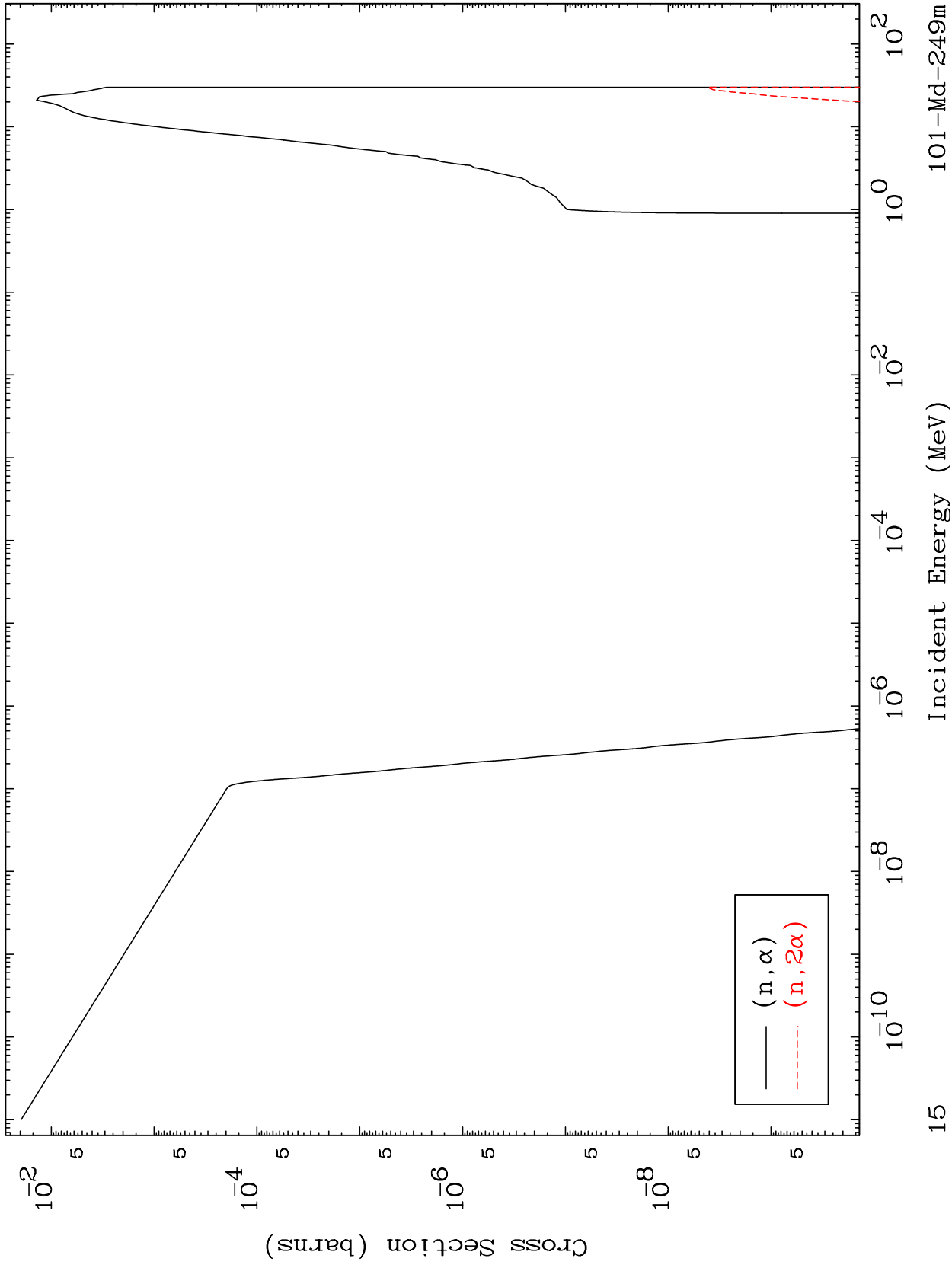
101-Md-249m



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(n, α) Levels
293 Kelvin Cross Sections

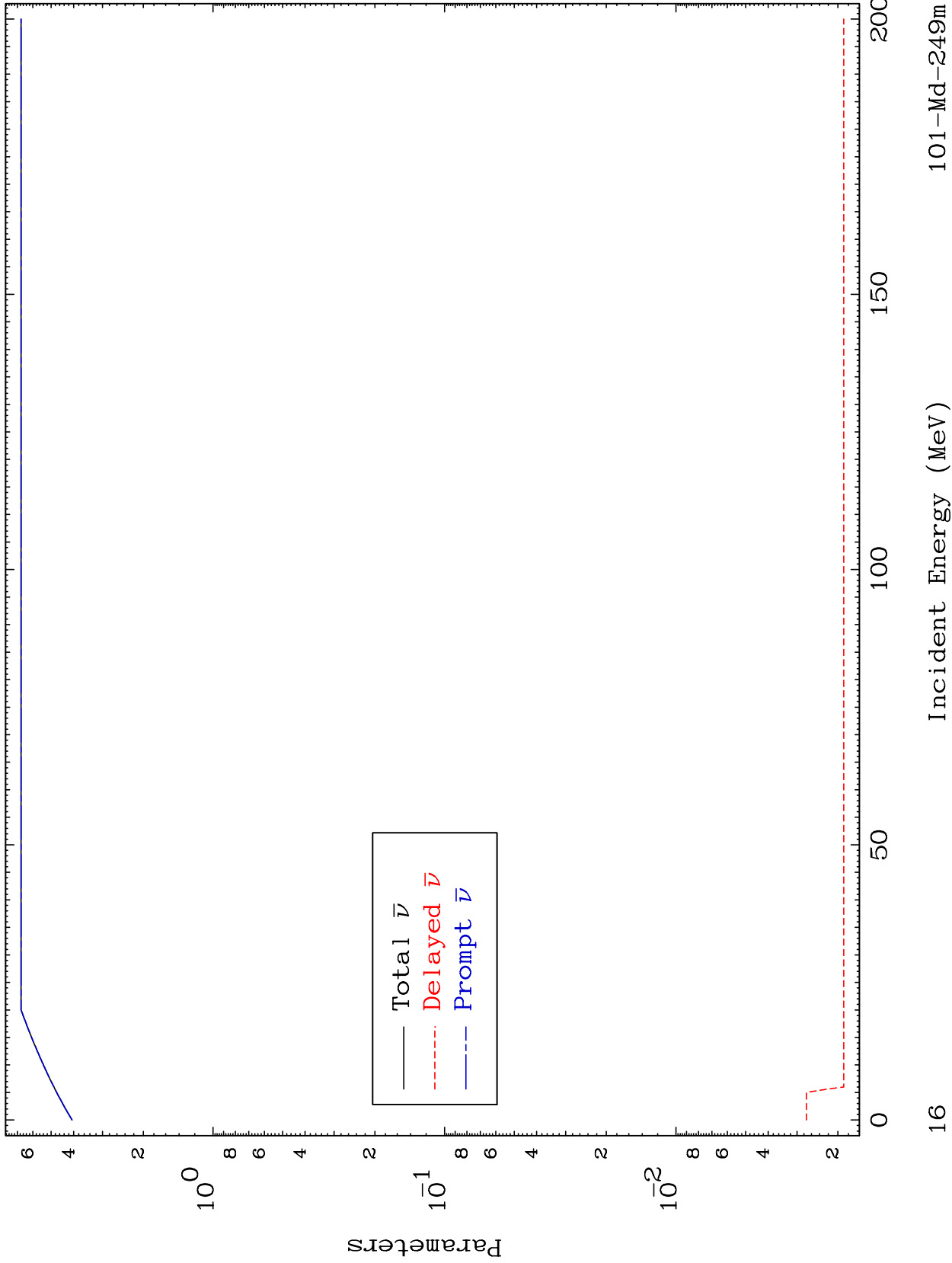
101-Md-249m



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Energy Release
Parameters

101-Md-249m



16

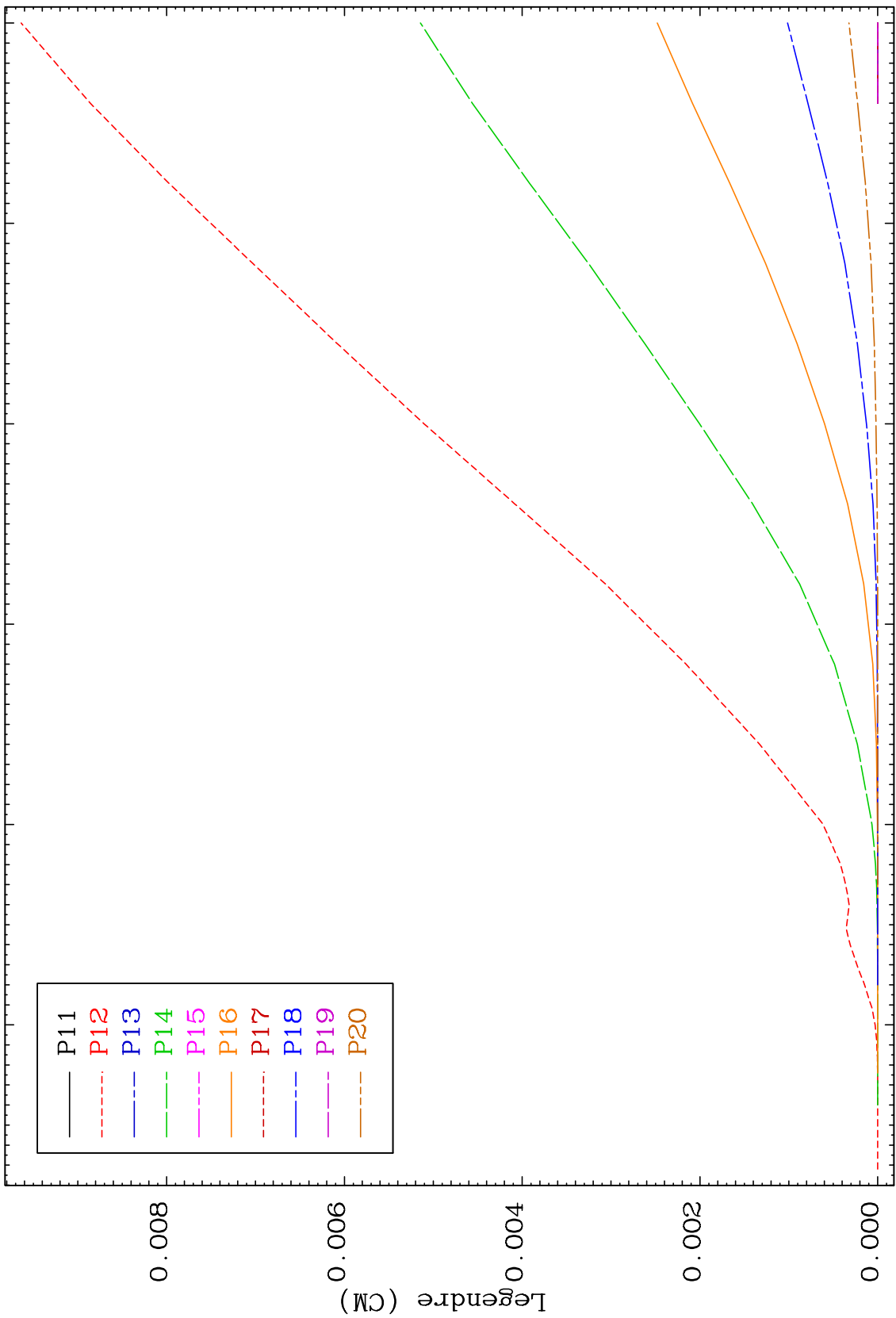
Incident Energy (MeV)

101-Md-249m

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Elastic Legendre Coefficients

101-Md-249m



18

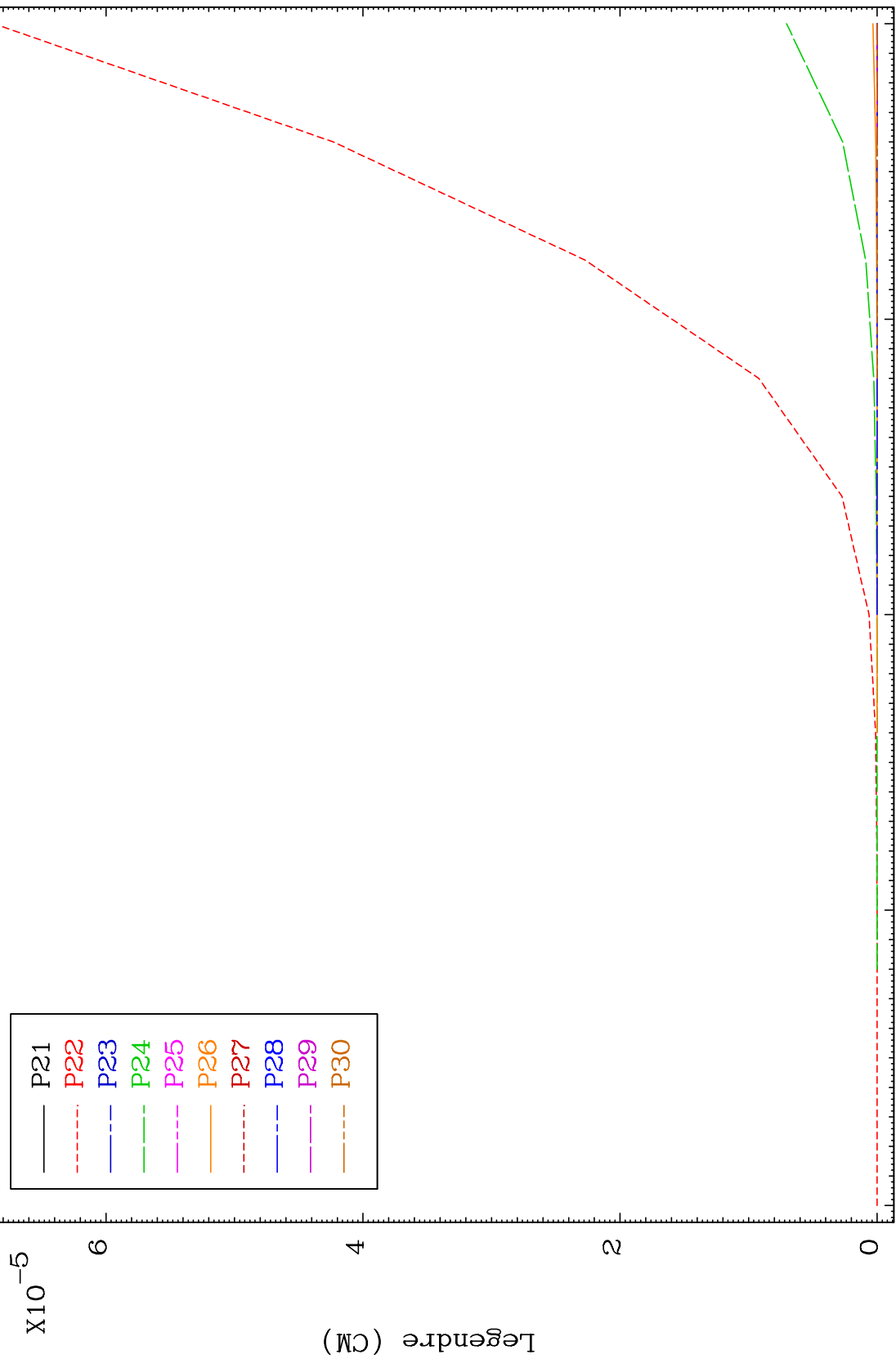
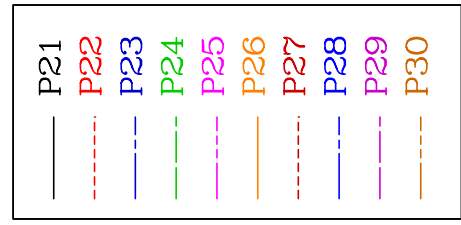
Incident Energy (MeV)

101-Md-249m

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Elastic
Legendre Coefficients

101-Md-249m



101-Md-249m

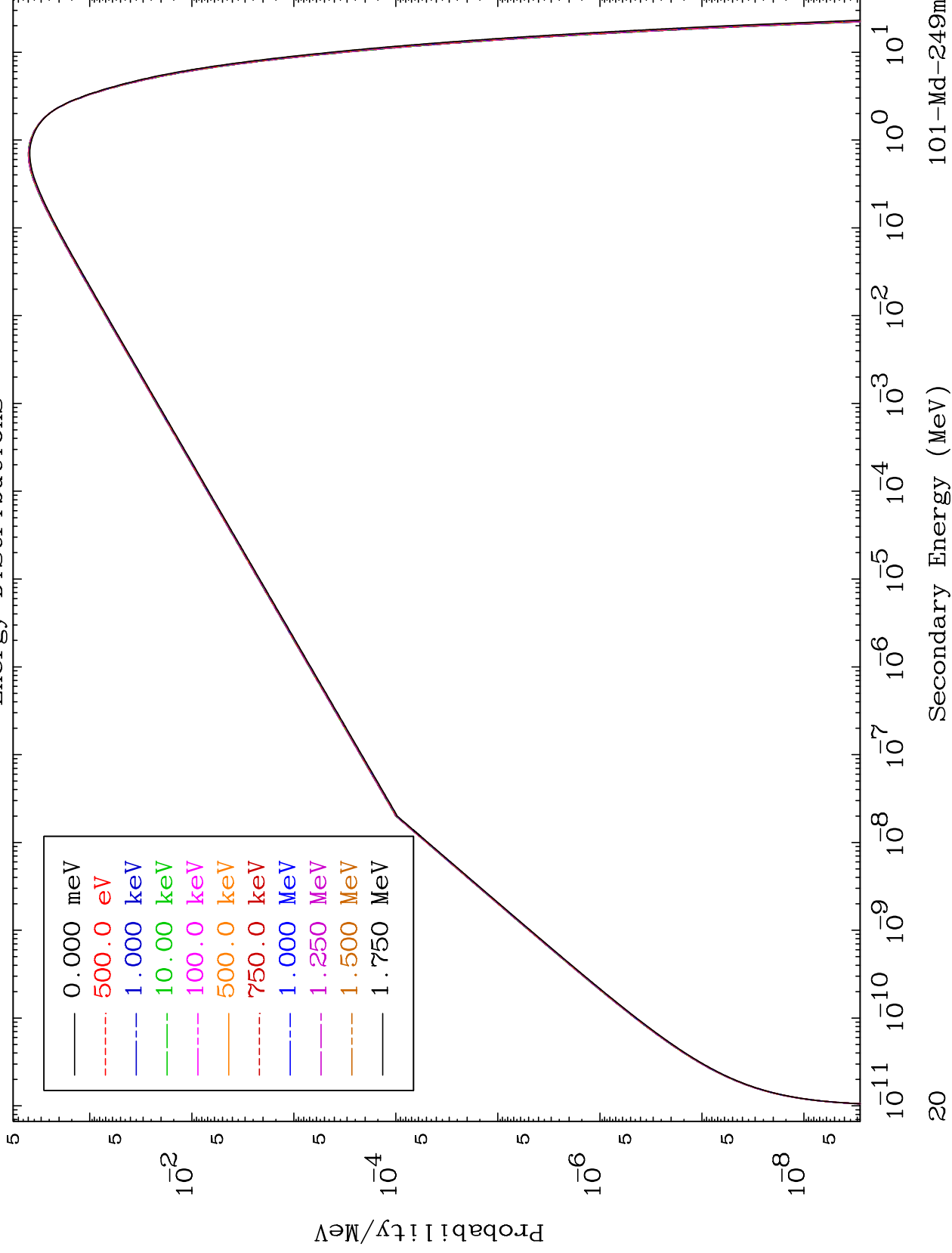
Incident Energy (MeV)

19

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Fission Energy Distributions

101-Md-249m



101-Md-249m

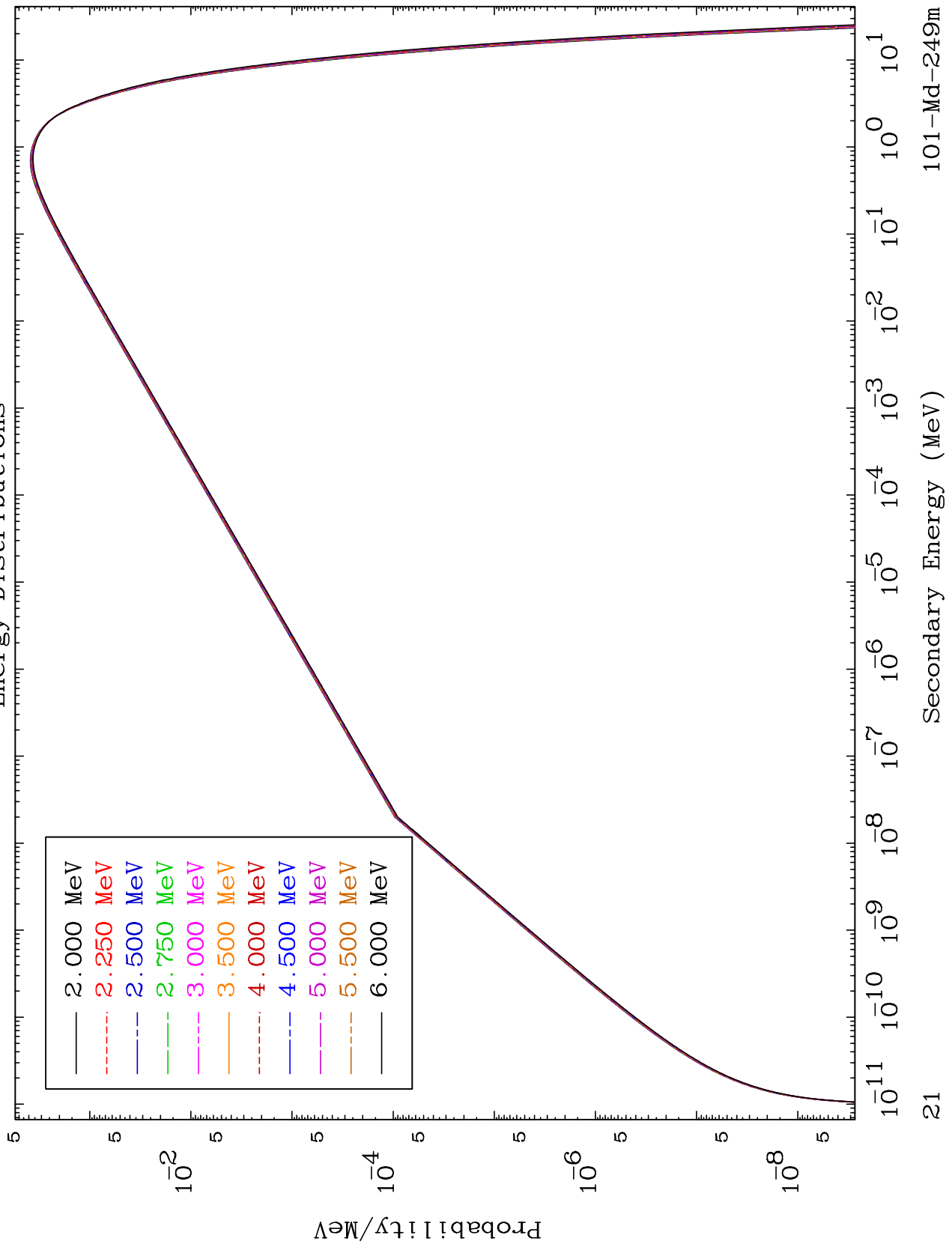
Secondary Energy (MeV)

20

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Fission Energy Distributions

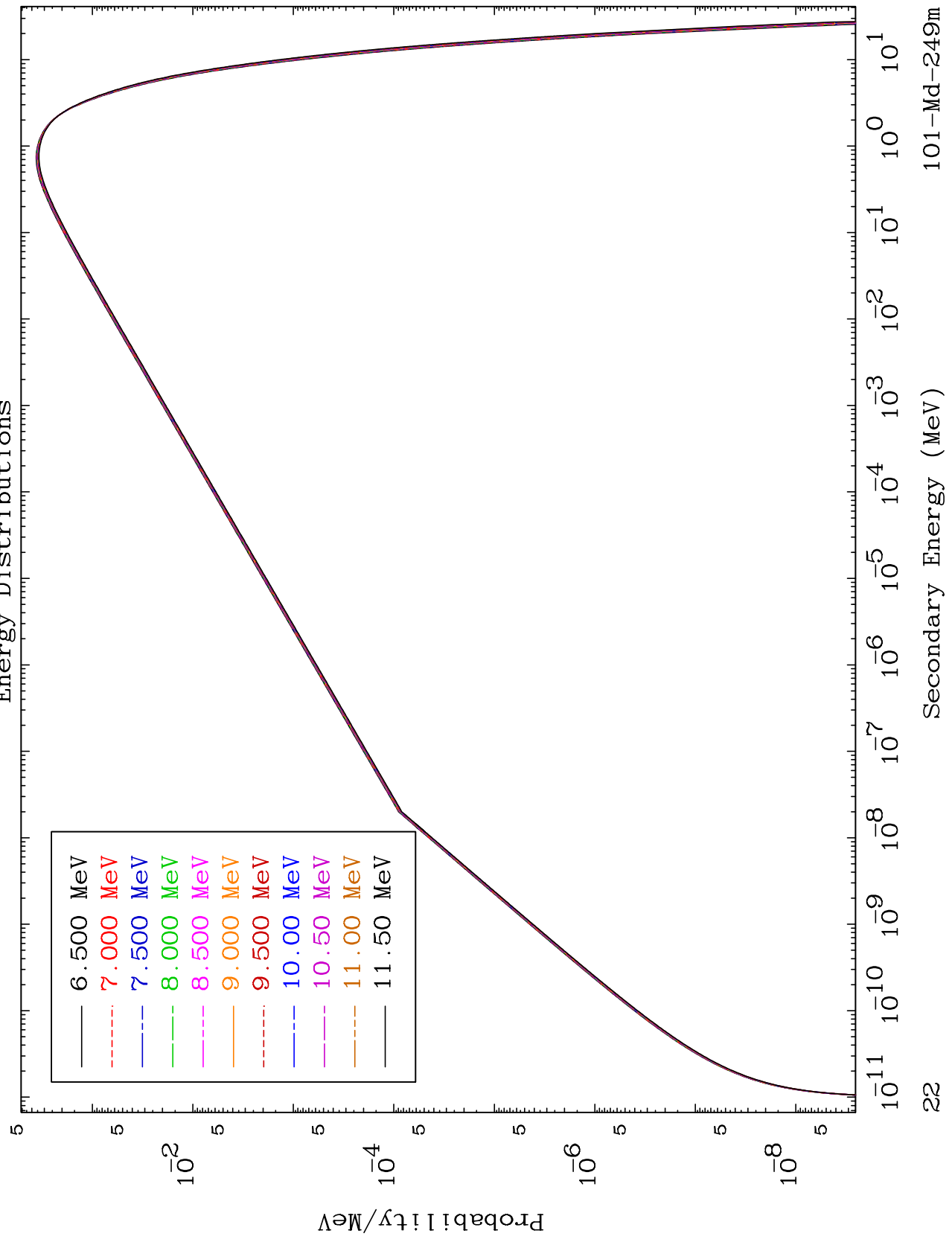
101-Md-249m



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Fission Energy Distributions

101-Md-249m



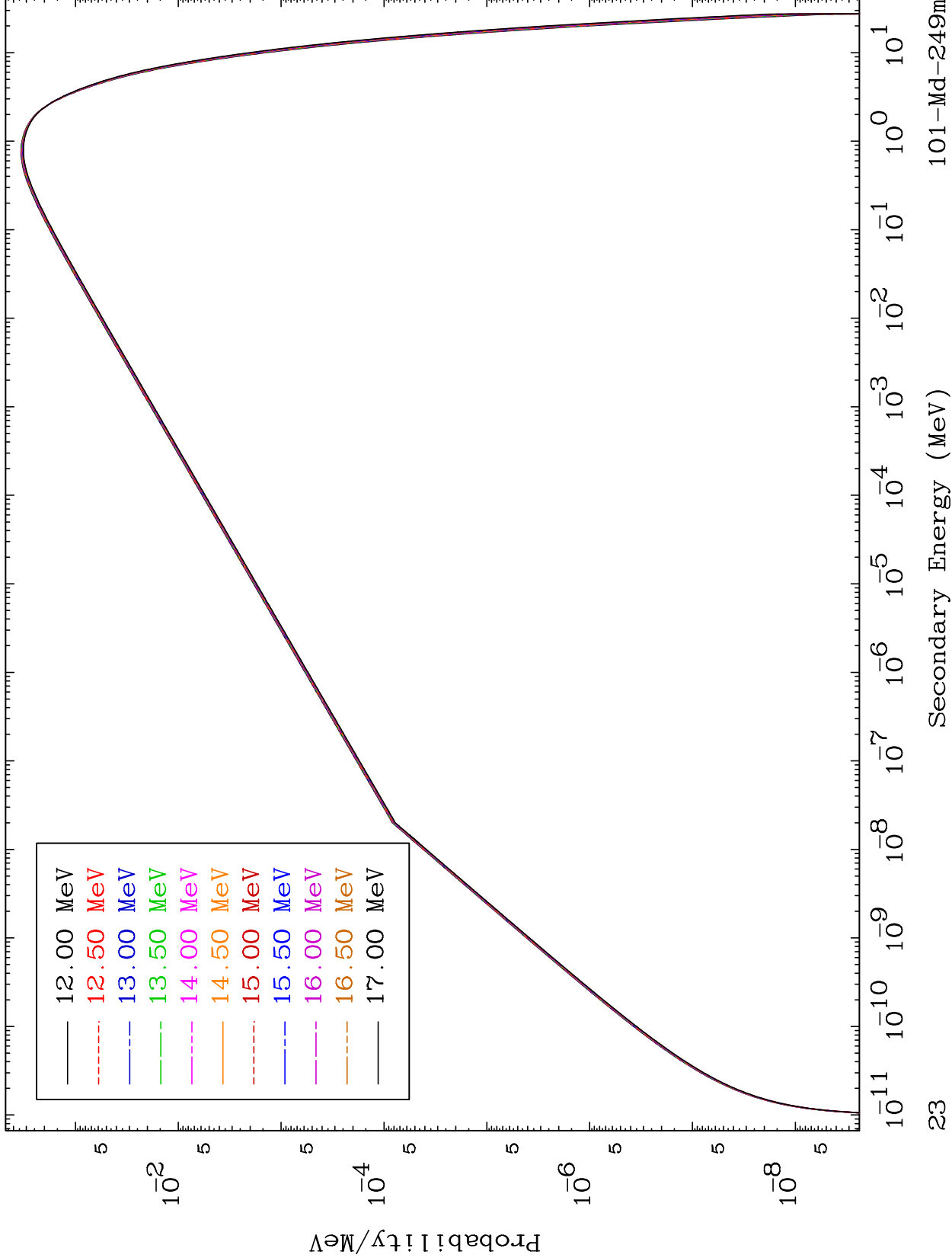
101-Md-249m

Secondary Energy (MeV)

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Fission Energy Distributions

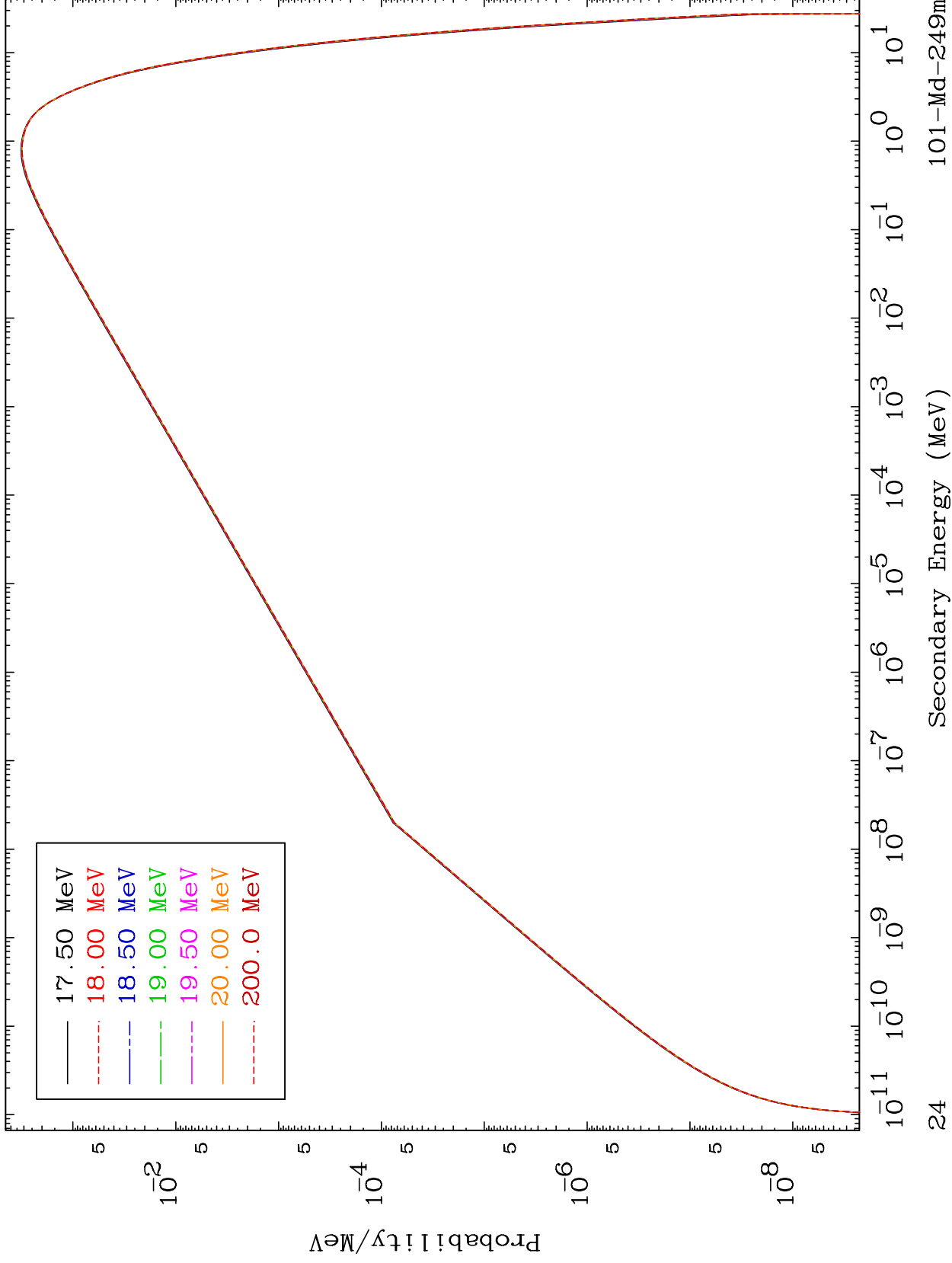
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Fission Energy Distributions

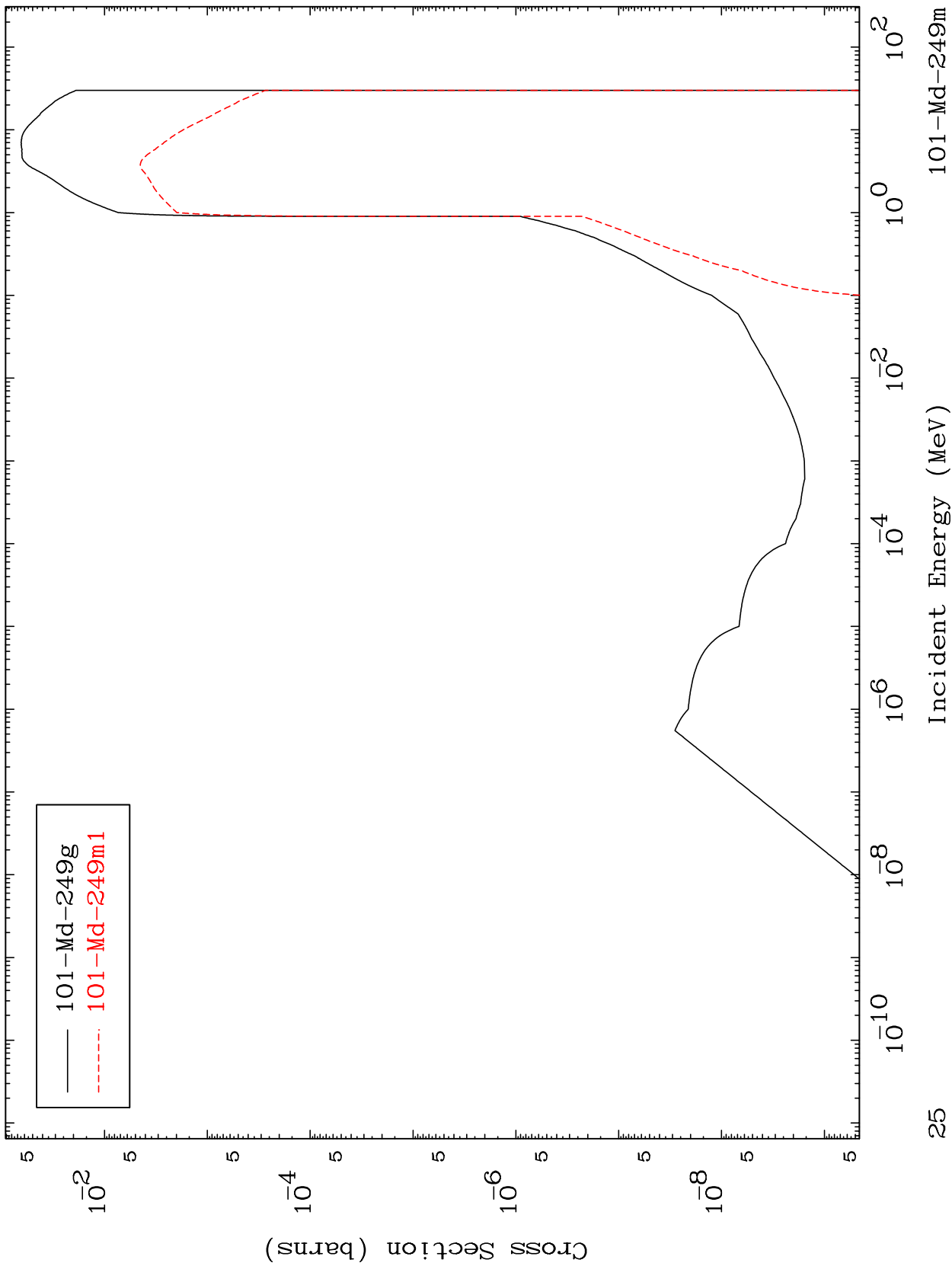
101-Md-249m



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Inelastic
Radionuclide Production Cross Section

101-Md-249m

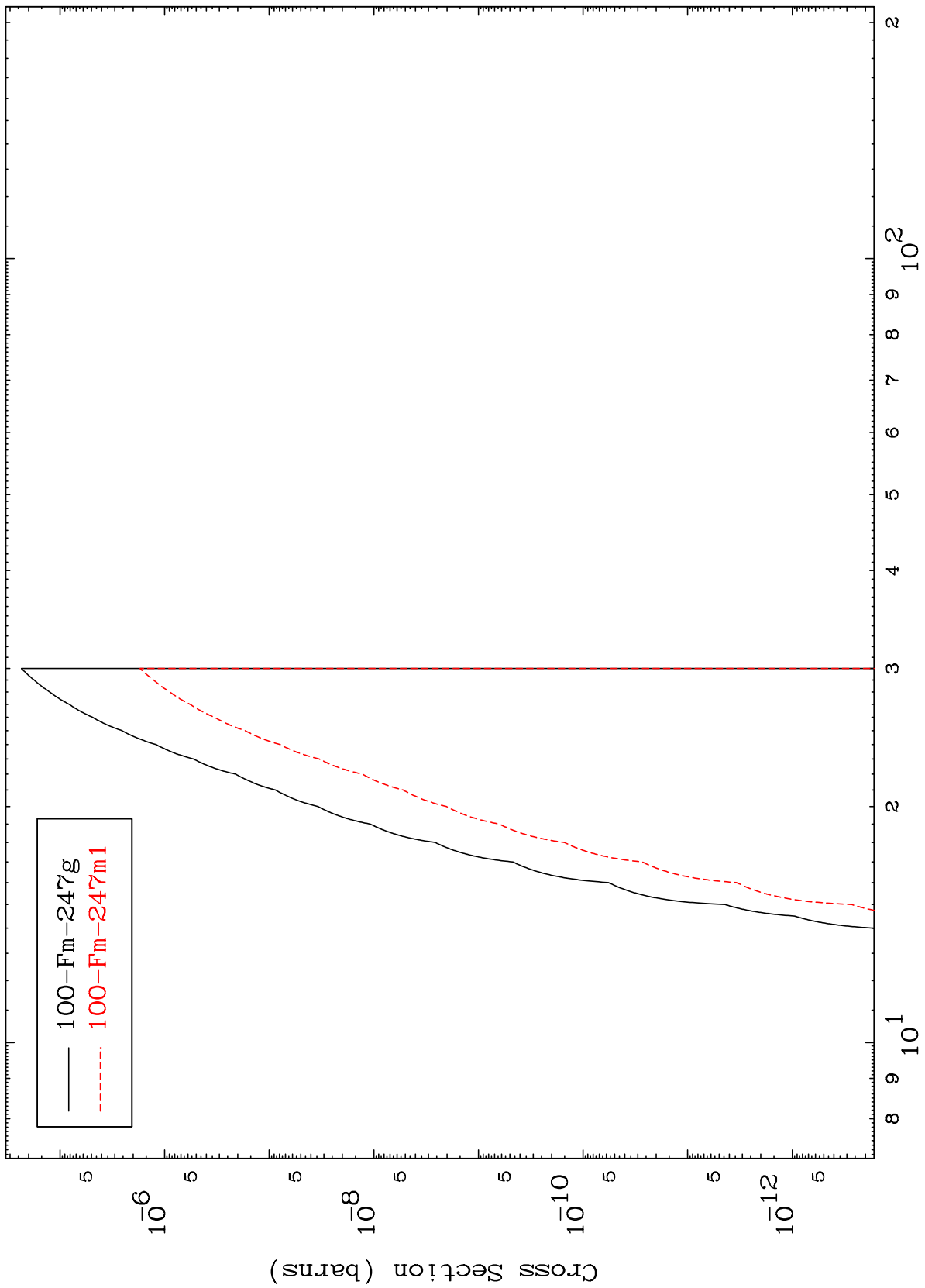


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(n,n') d

101-Md-249m

Radionuclide Production Cross Section



26

Incident Energy (MeV)

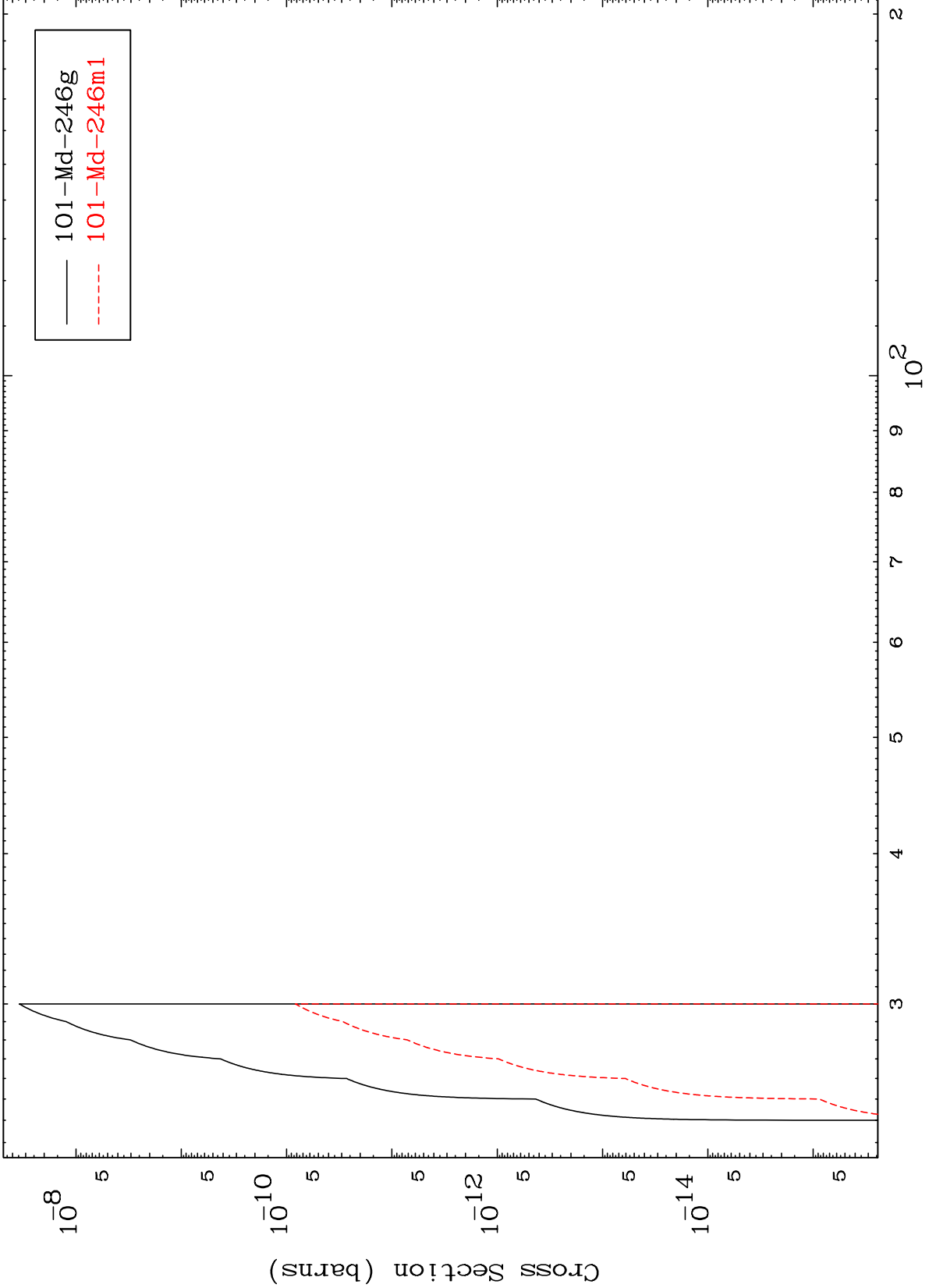
101-Md-249m

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(n,4n)

101-Md-249m

Radionuclide Production Cross Section



27

Incident Energy (MeV)

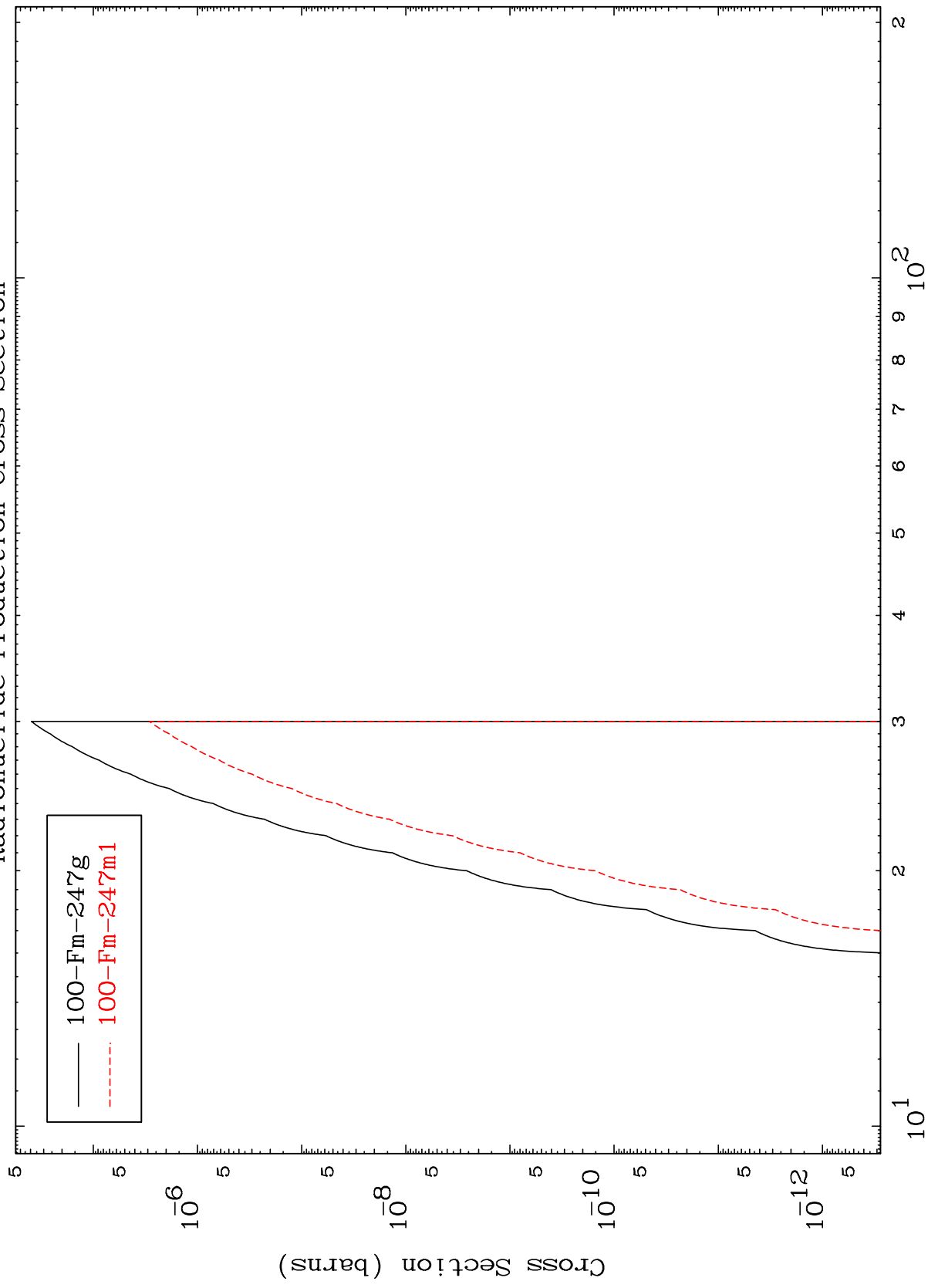
101-Md-249m

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(n,2n) p

101-Md-249m

Radionuclide Production Cross Section



Incident Energy (MeV)

101-Md-249m

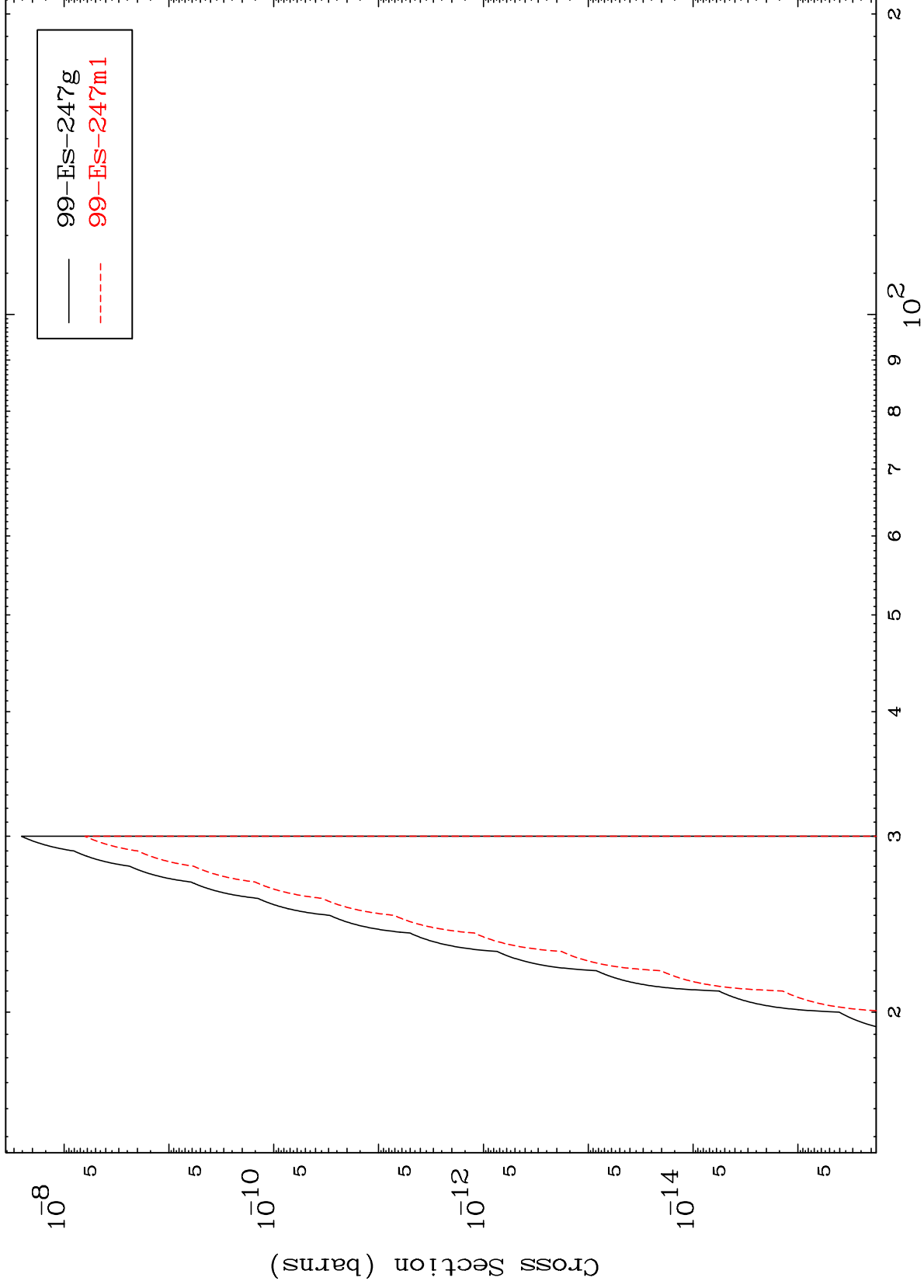
28

MAT 9955

(n,2n) p

101-Md-249m

Radionuclide Production Cross Section



29

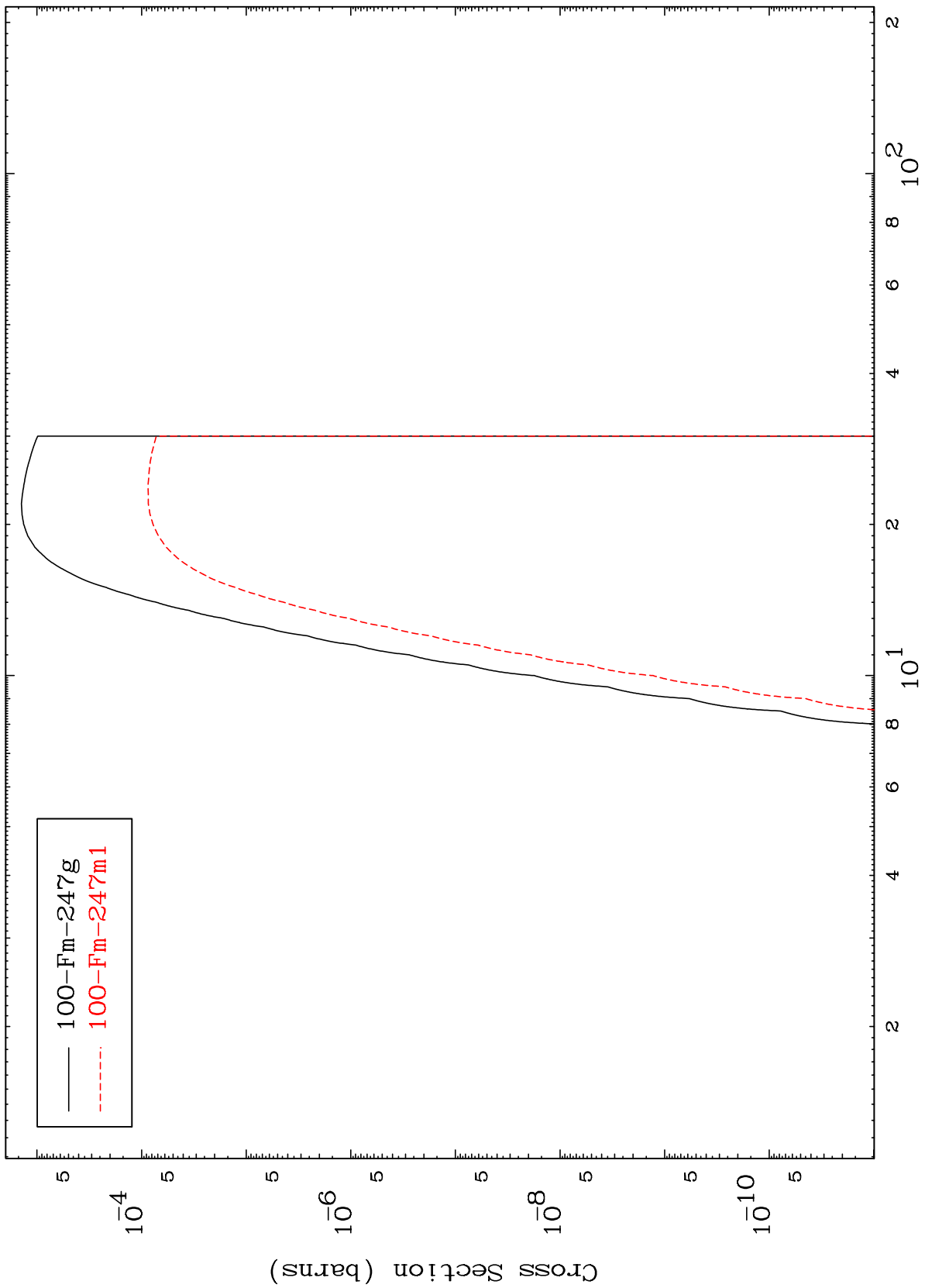
Incident Energy (MeV)

101-Md-249m

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101-Md-249m

(n, t)
Radionuclide Production Cross Section



101-Md-249m

Incident Energy (MeV)

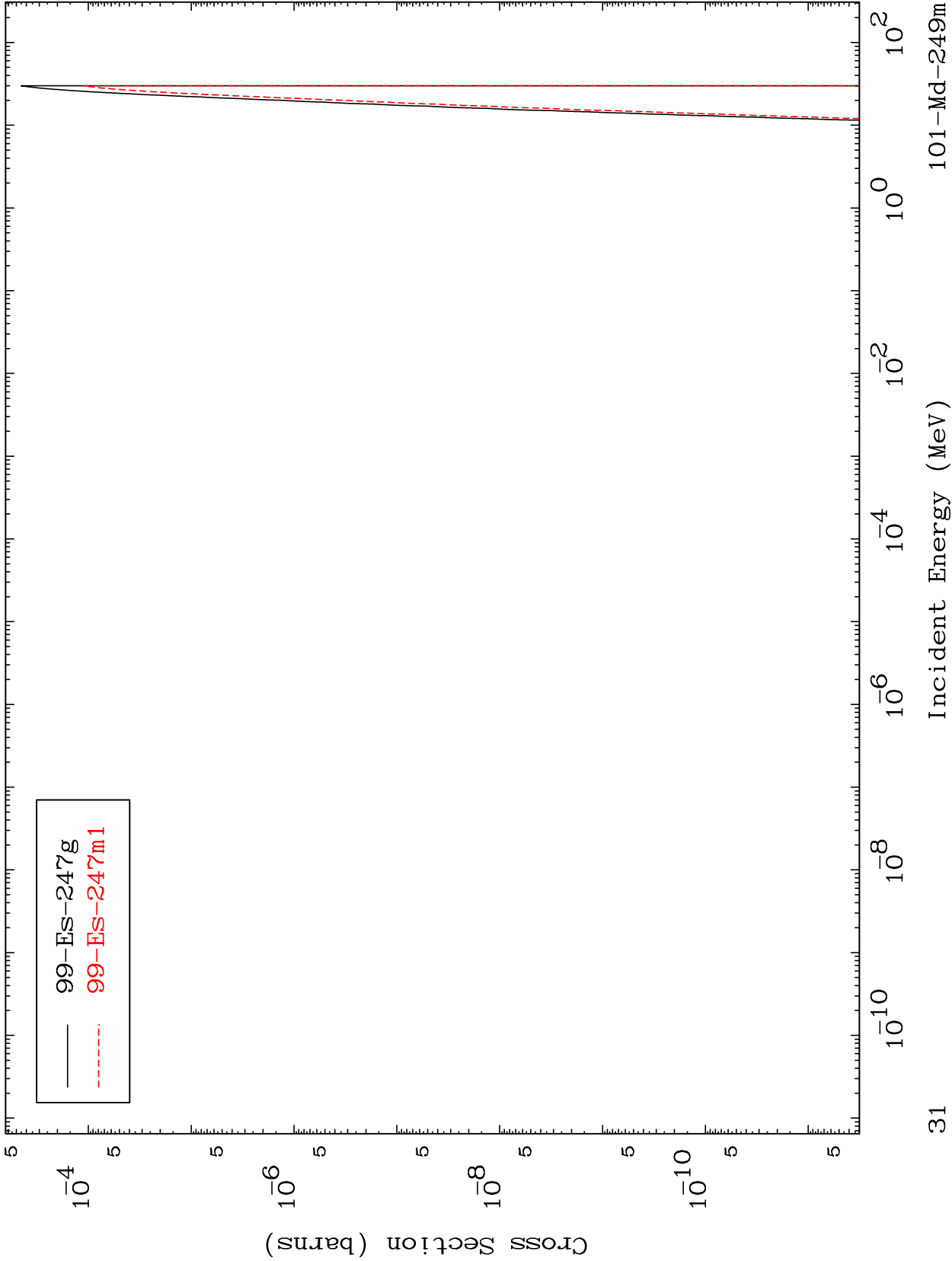
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(n,He-3)

101-Md-249m

Radionuclide Production Cross Section



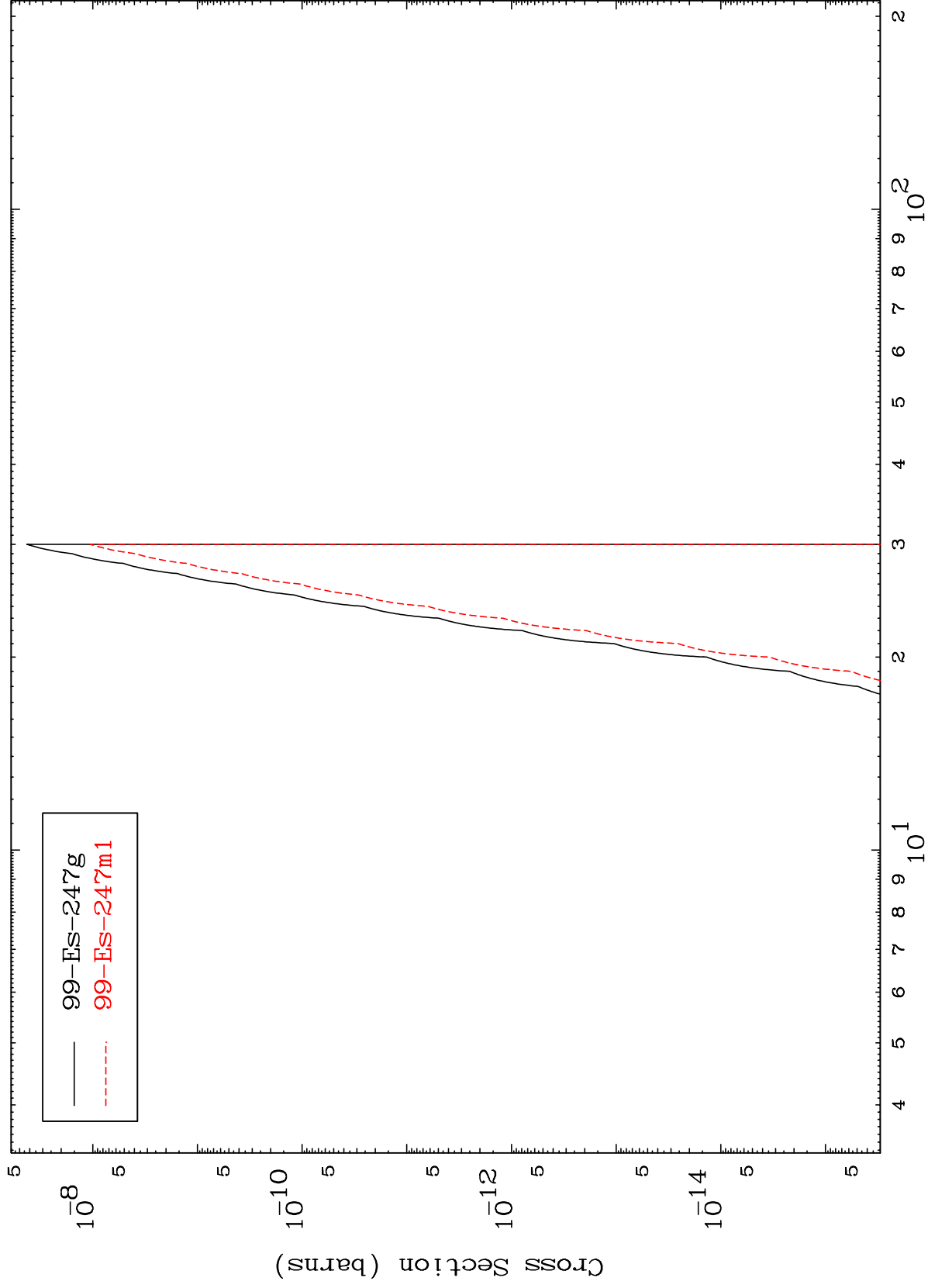
99-Es-247g
99-Es-247m1

MAT 9955

(n,p) d

101-Md-249m

Radionuclide Production Cross Section



32

Incident Energy (MeV)

101-Md-249m