

Program EVALPLOT
(Version 2021-1)

by

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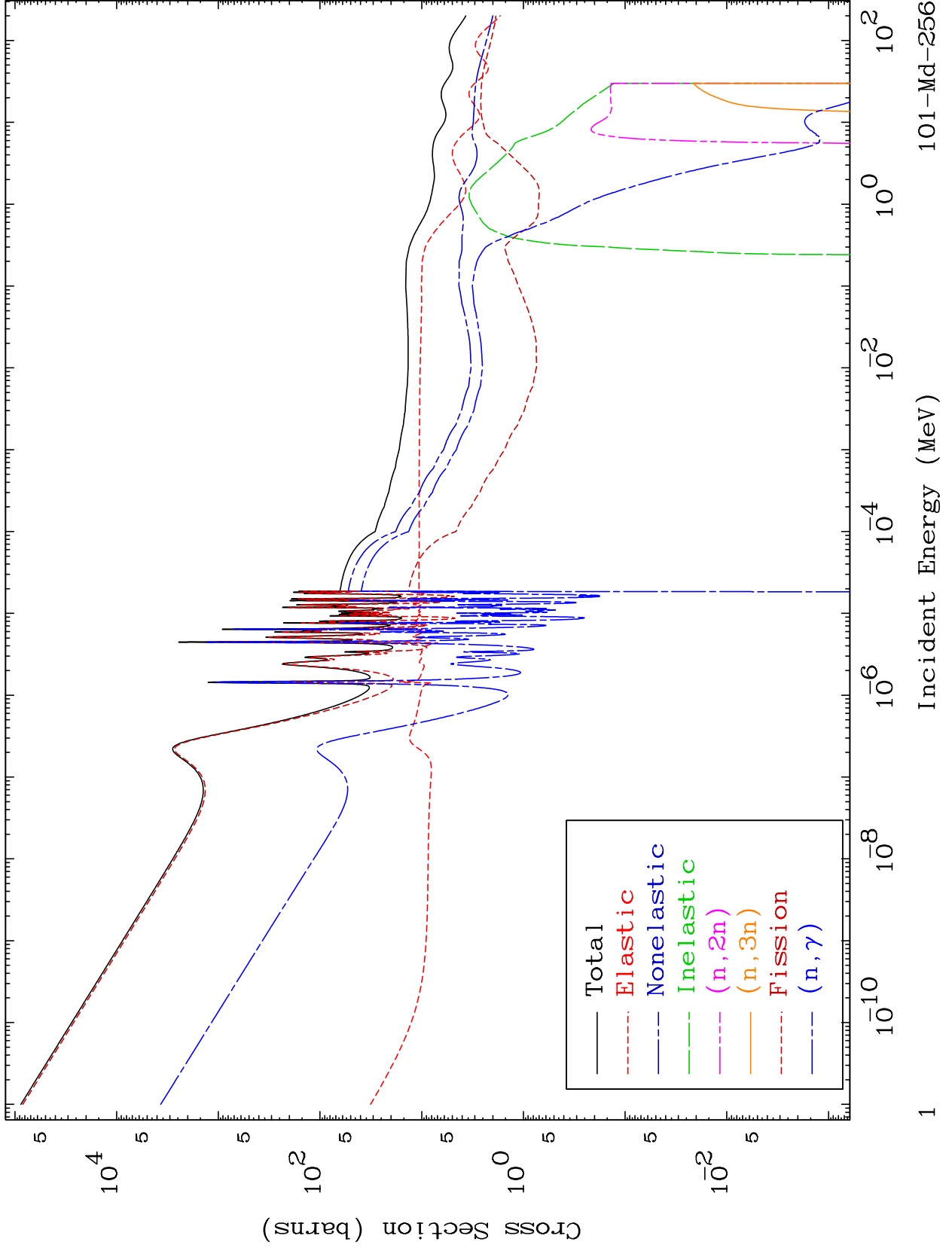
Web: redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9957

Neutron Major
293 Kelvin Cross Sections

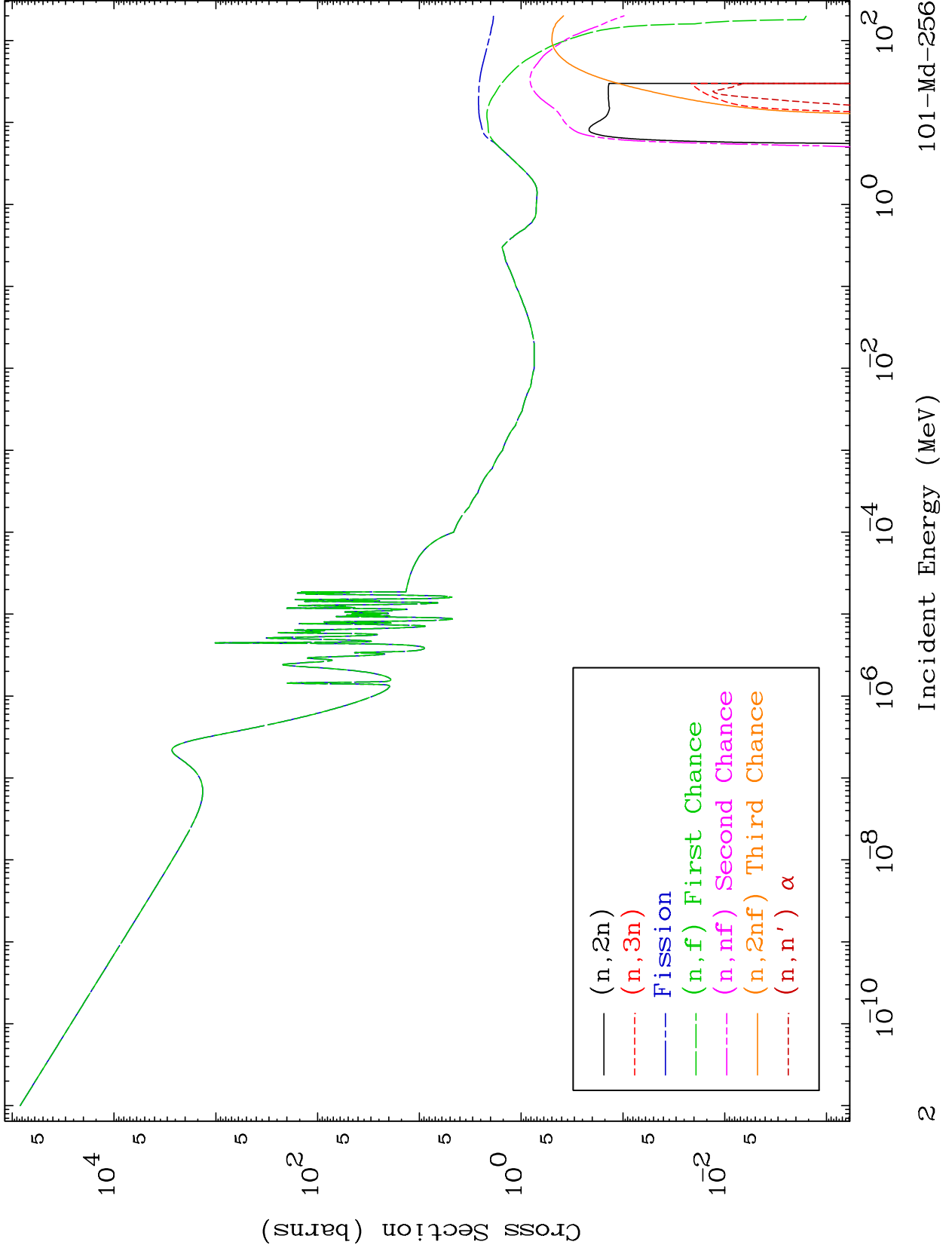
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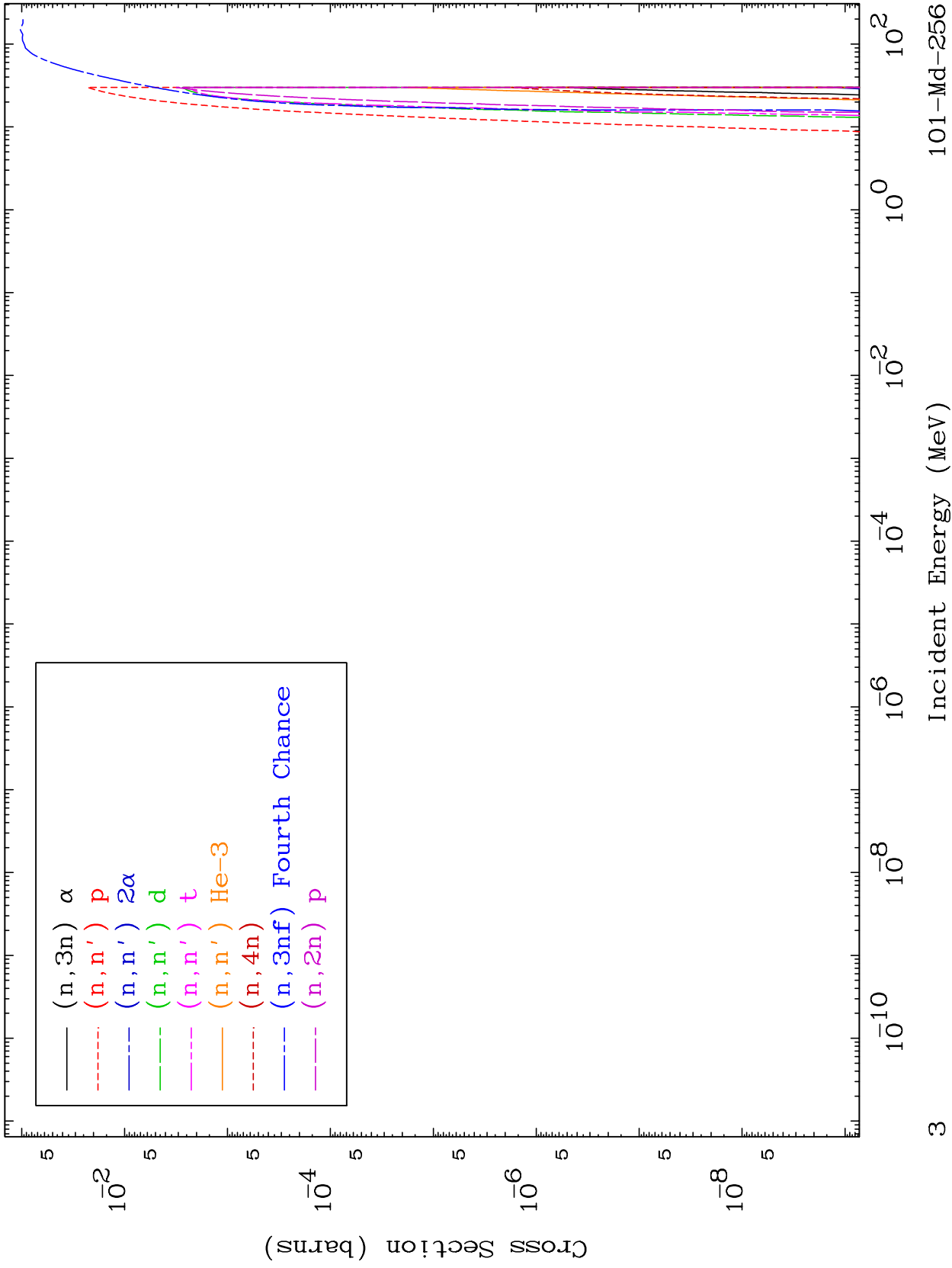


MAT 9957

Neutron Absorption
293 Kelvin Cross Sections

101-Md-256

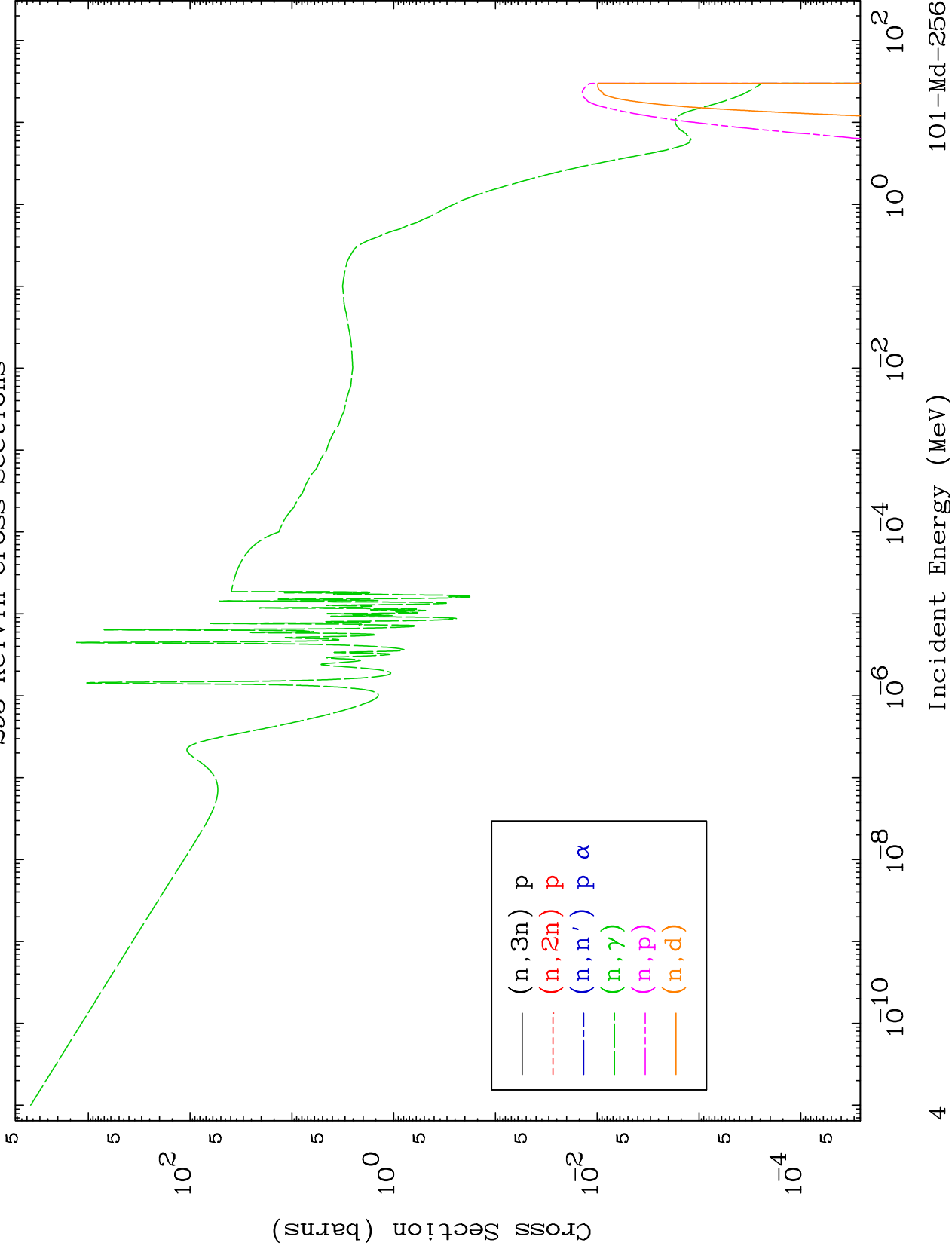




MAT 9957

Neutron Absorption
293 Kelvin Cross Sections

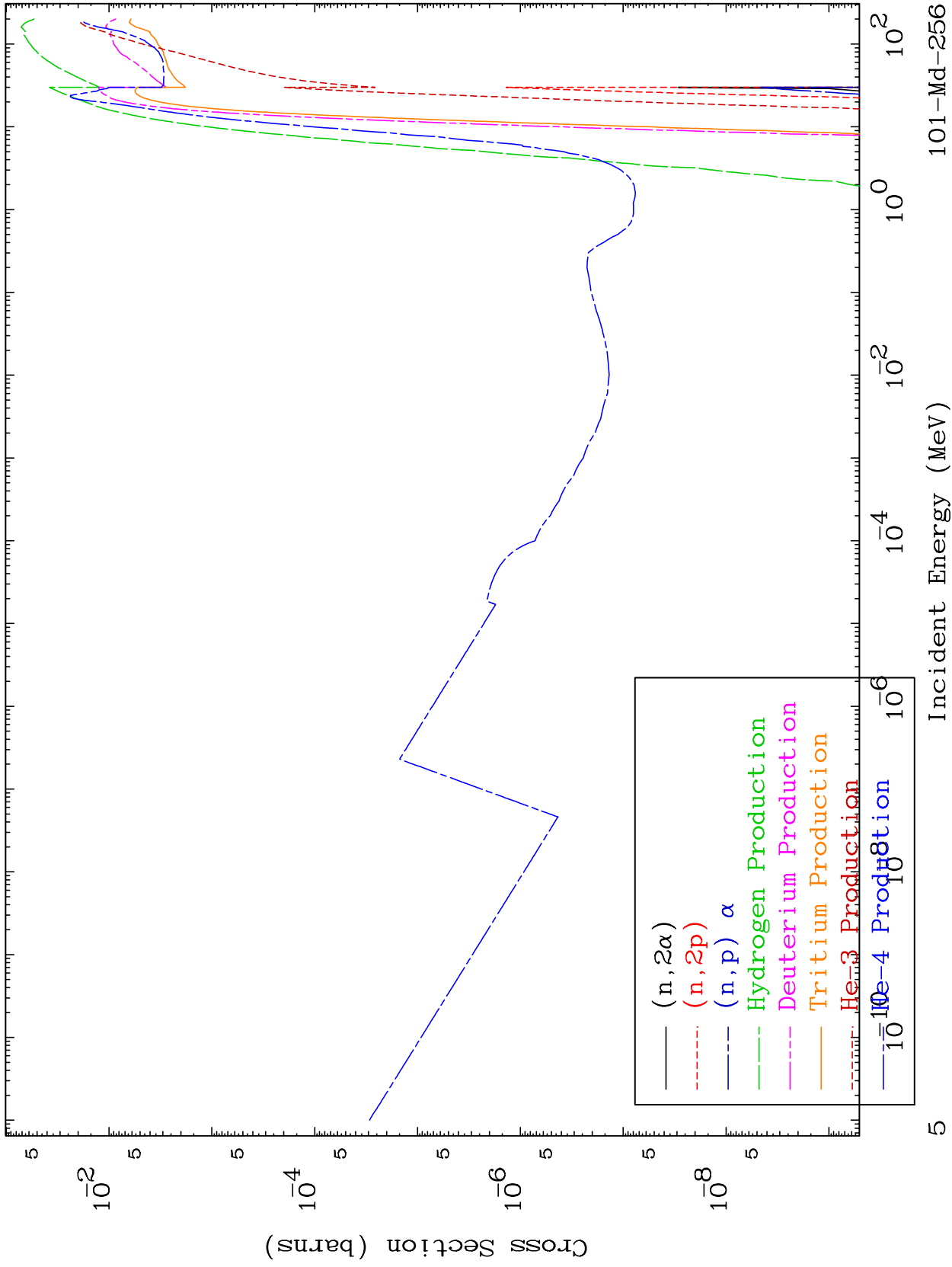
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MAT 9957

Neutron Absorption
293 Kelvin Cross Sections

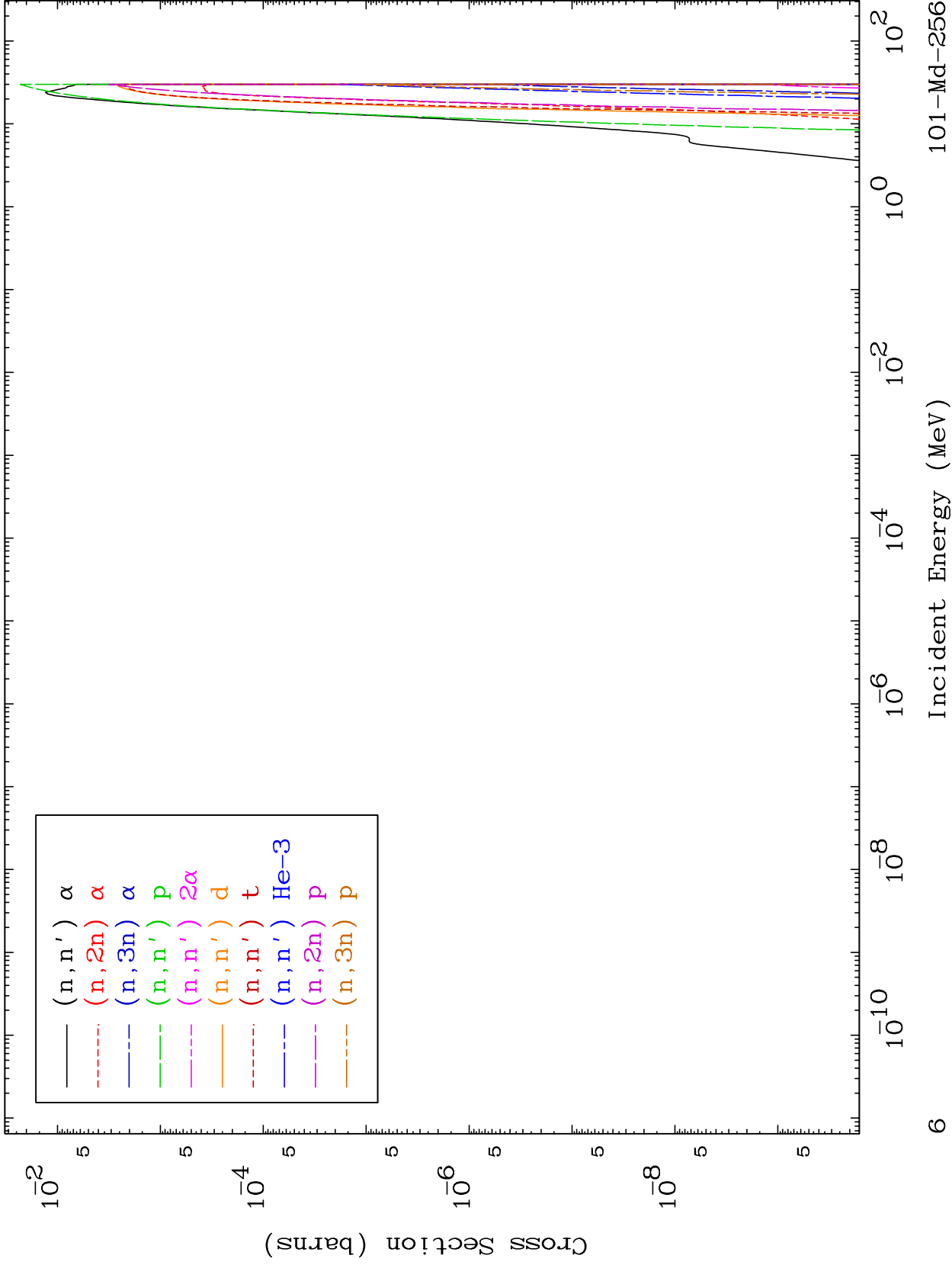
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MAT 9957

Charged Particle
293 Kelvin Cross Sections

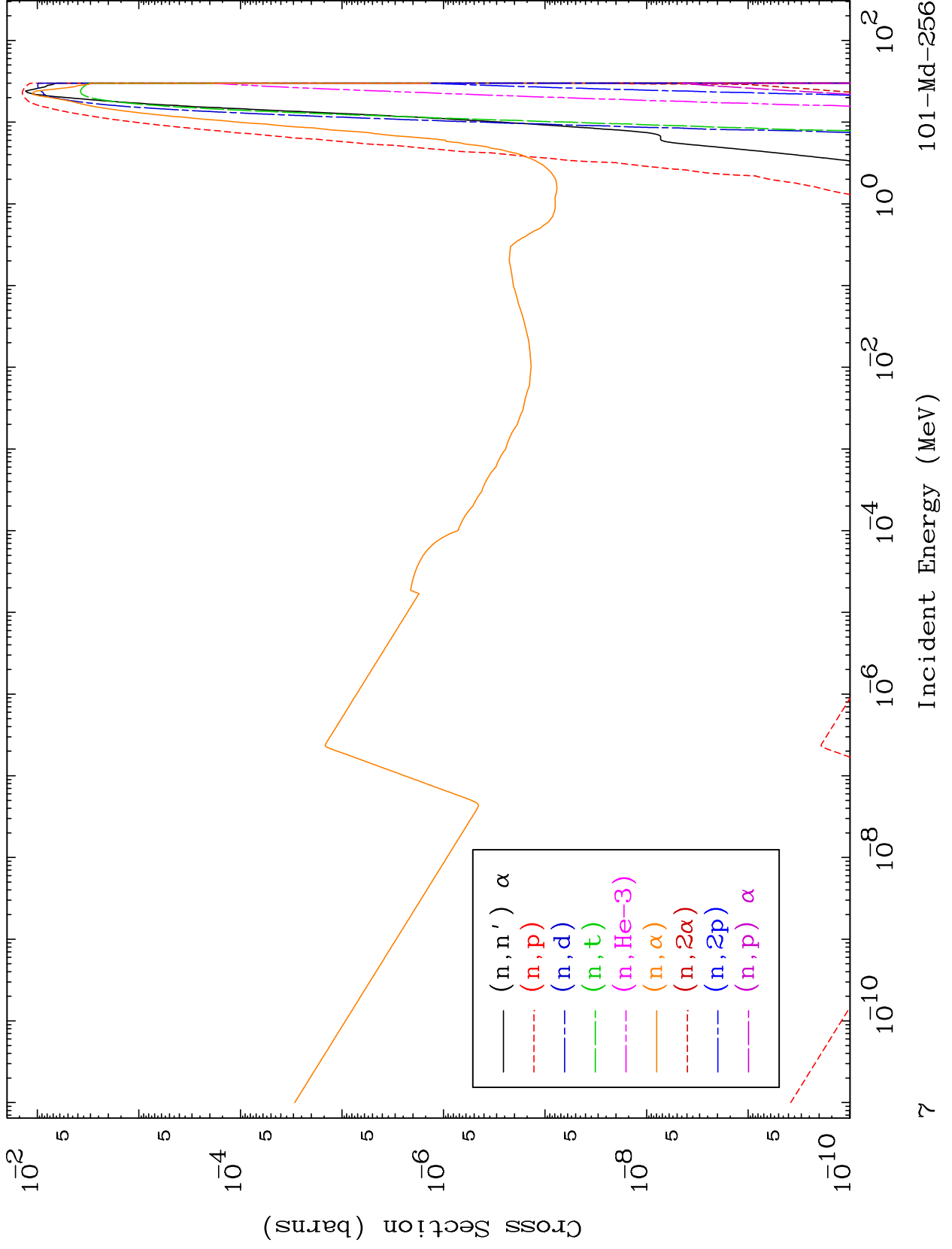
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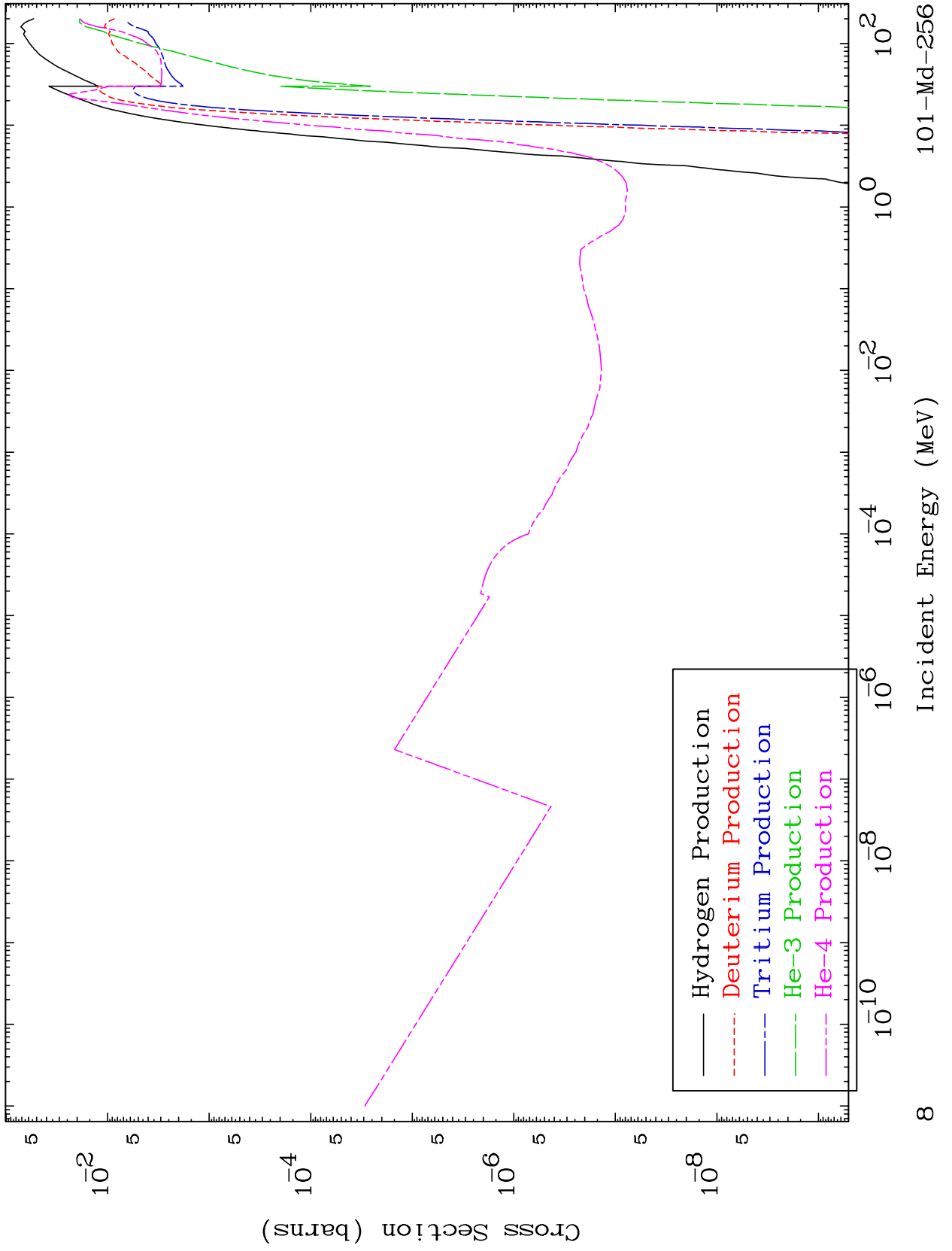


MAT 9957

Charged Particle
293 Kelvin Cross Sections

101-Md-256

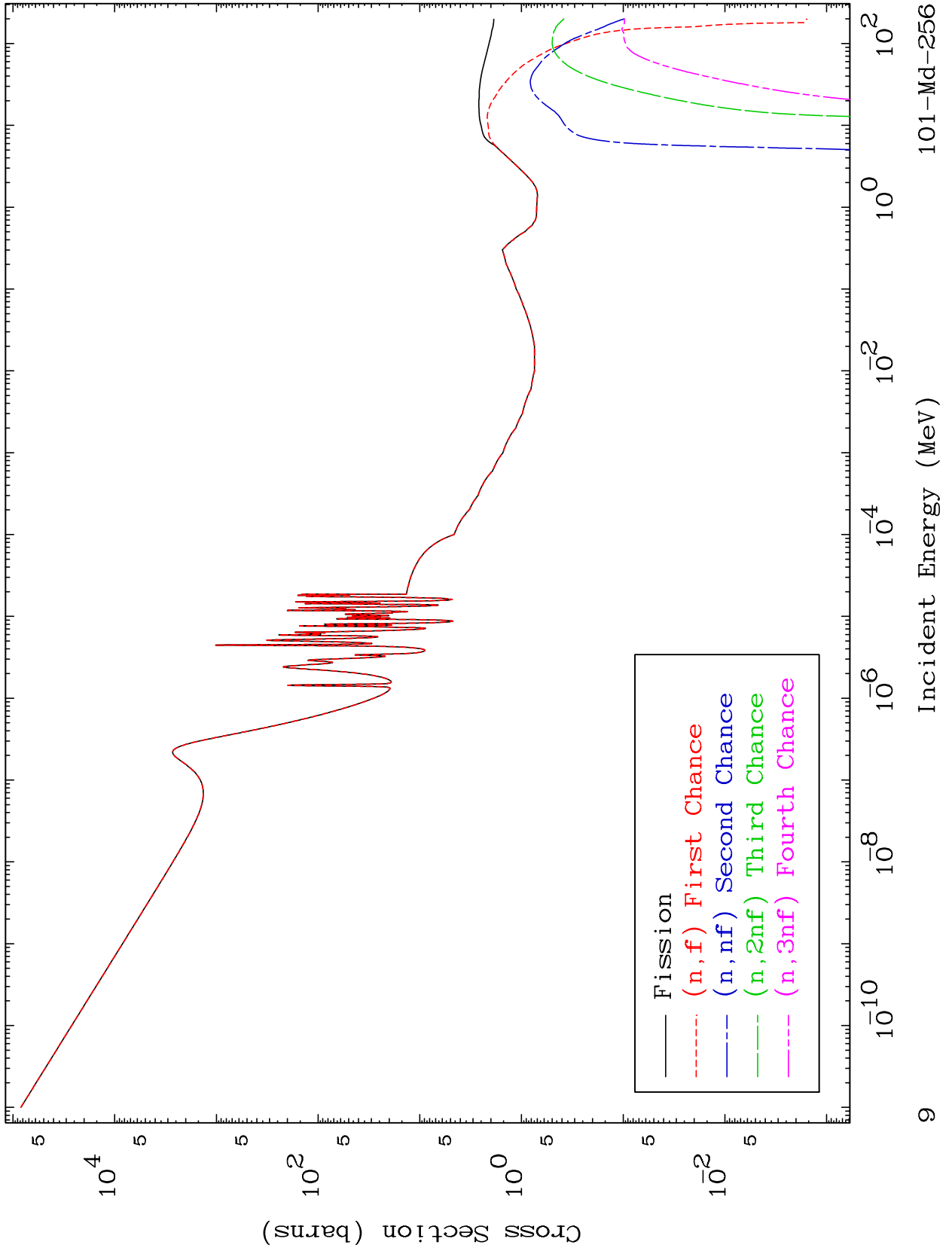




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Fission
293 Kelvin Cross Sections

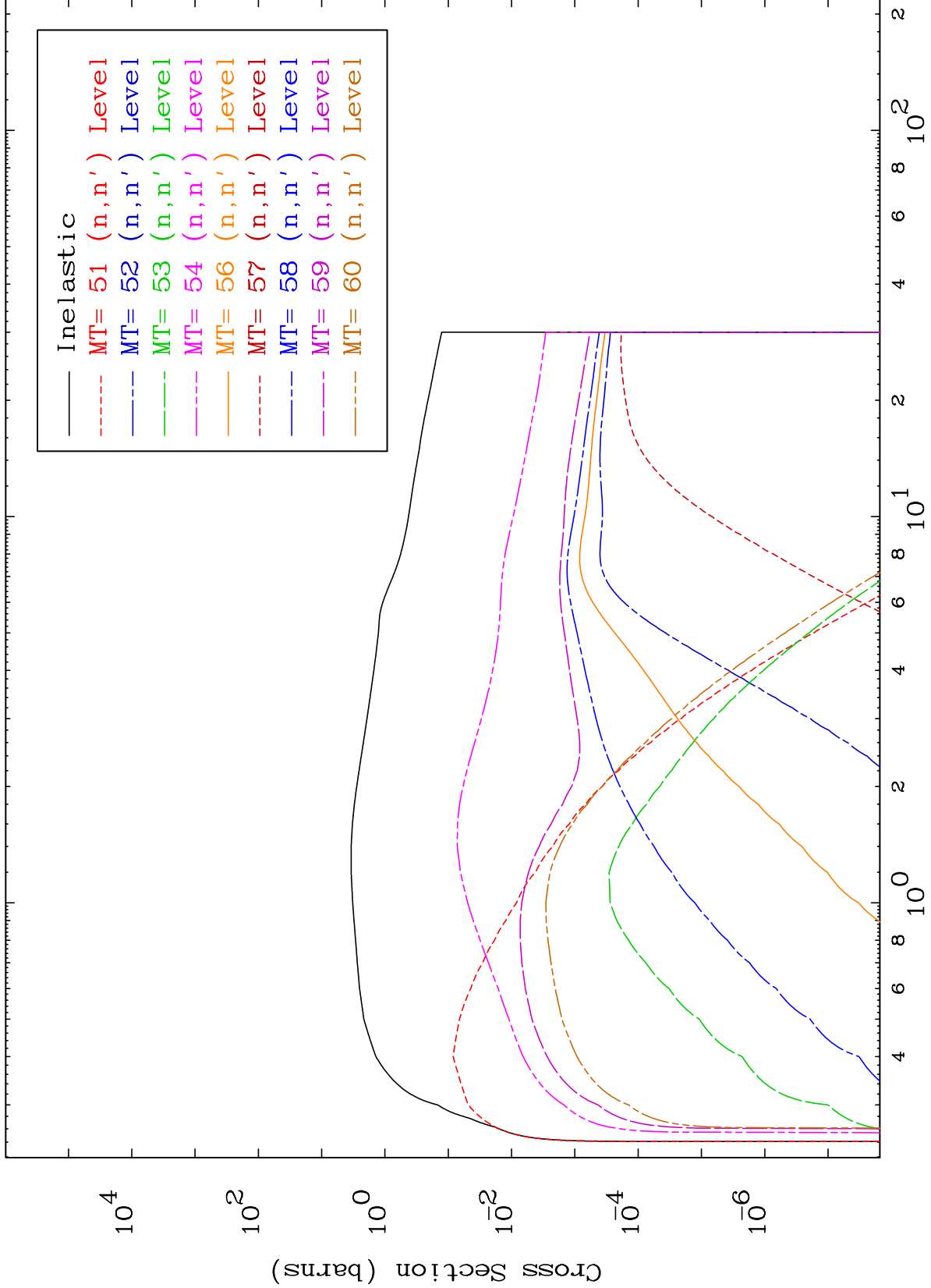
101-Md-256



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(n,n') Levels
293 Kelvin Cross Sections

101-Md-256



10

Incident Energy (MeV)

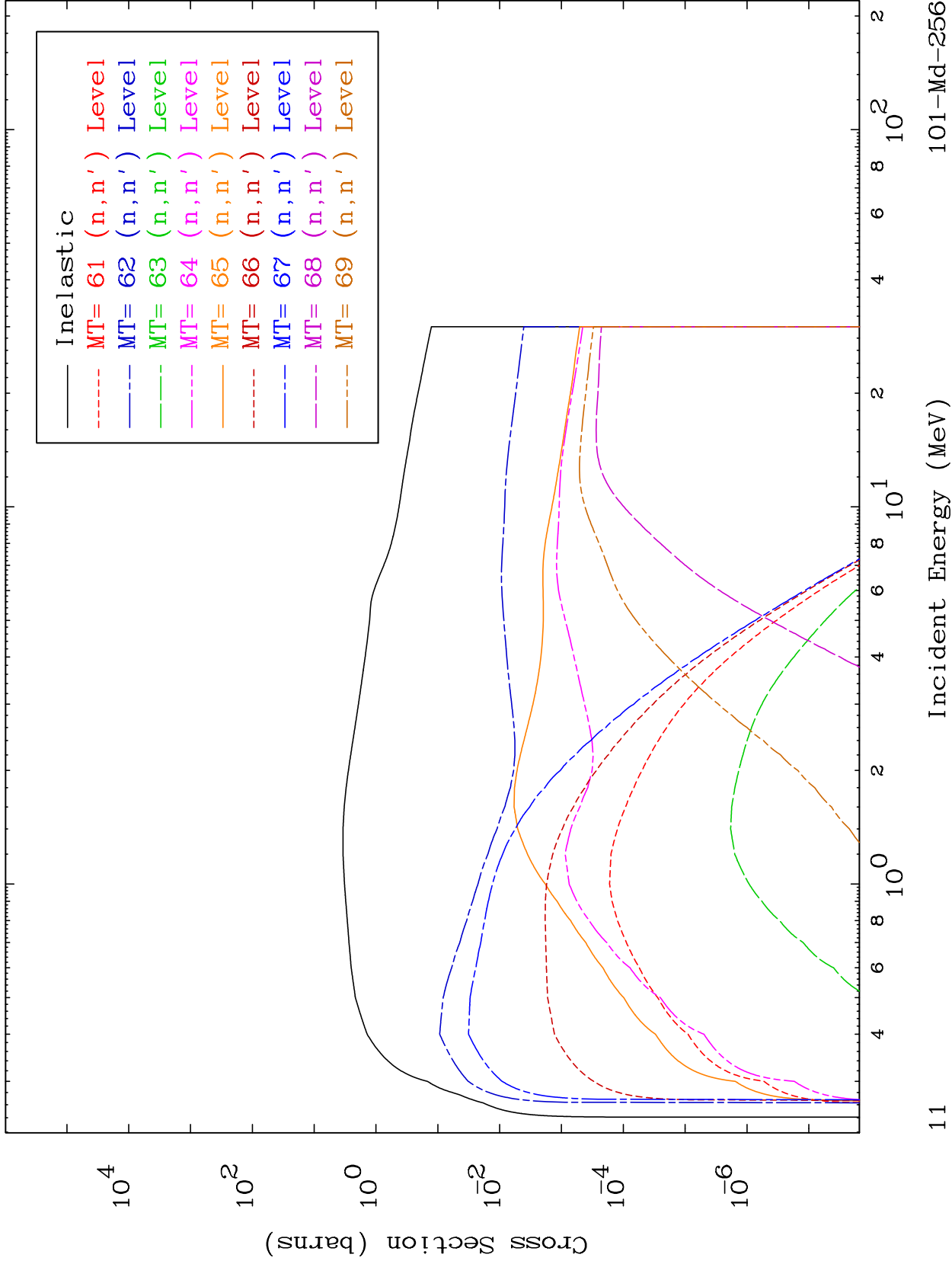
101-Md-256

MAT 9957

(n,n') Levels

101-Md-256

293 Kelvin Cross Sections



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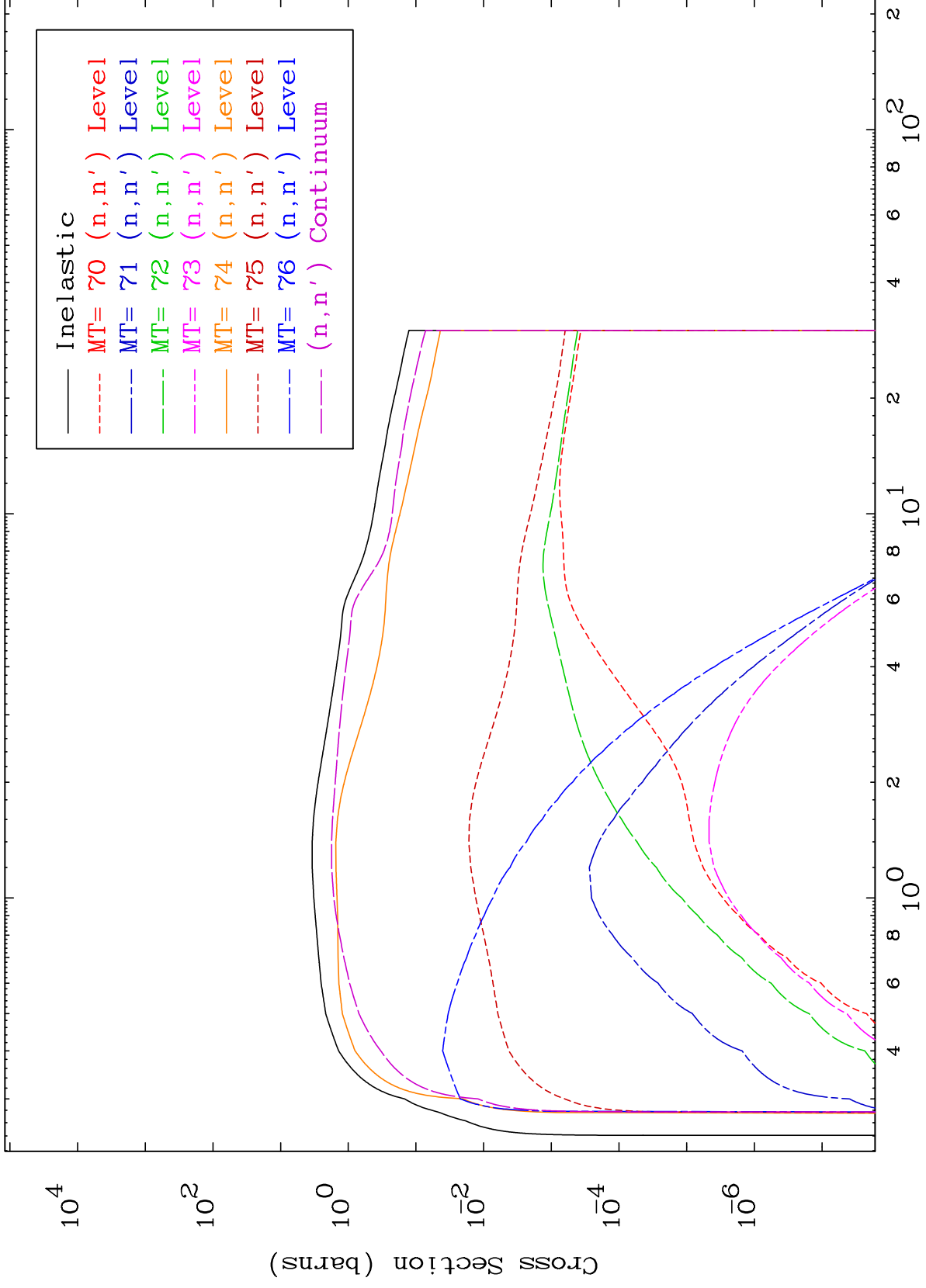
Incident Energy (MeV)

101-Md-256

MAT 9957

(n,n') Levels
293 Kelvin Cross Sections

101-Md-256



12

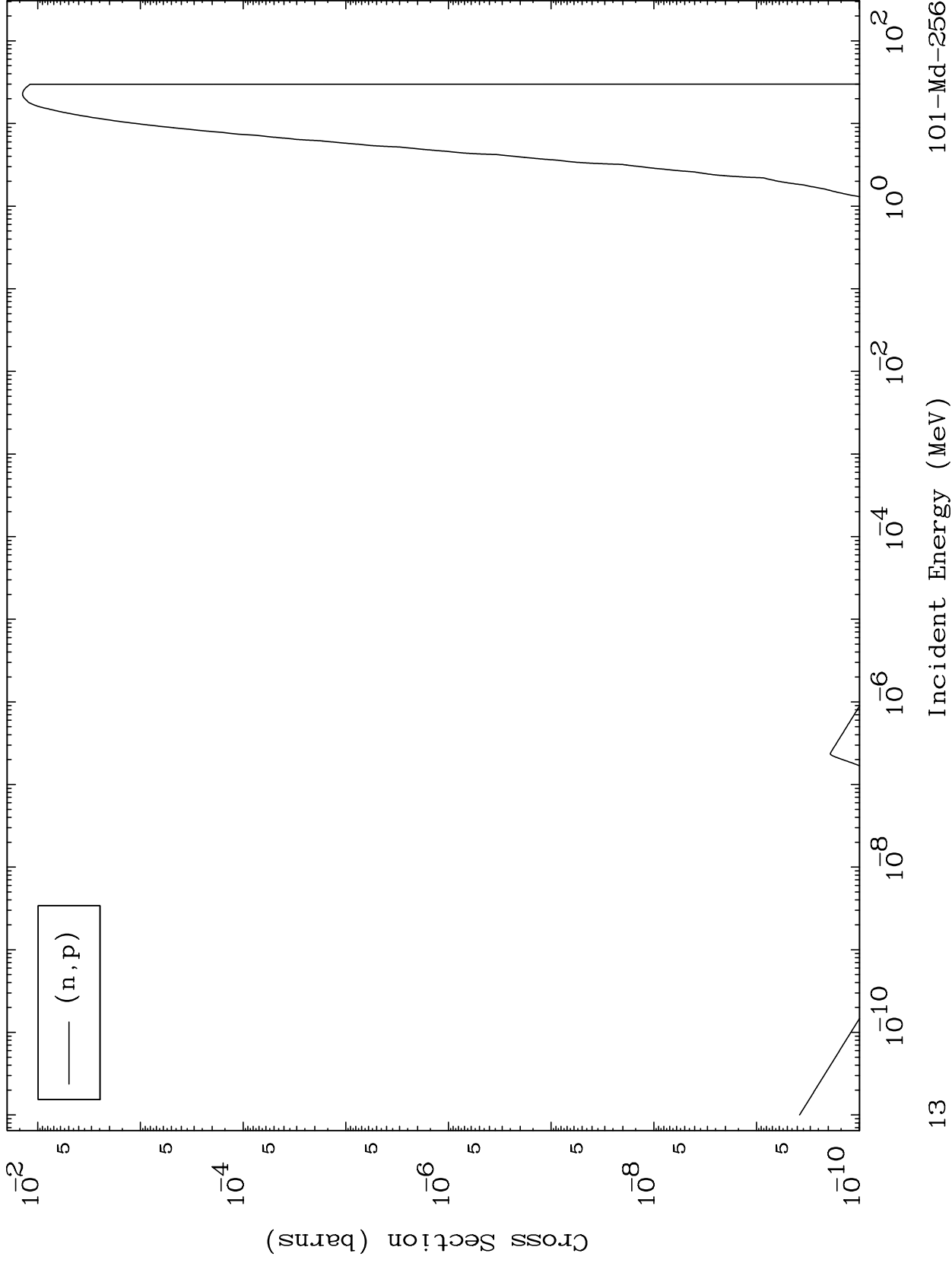
Incident Energy (MeV)

101-Md-256

MAT 9957

(n,p) Levels
293 Kelvin Cross Sections

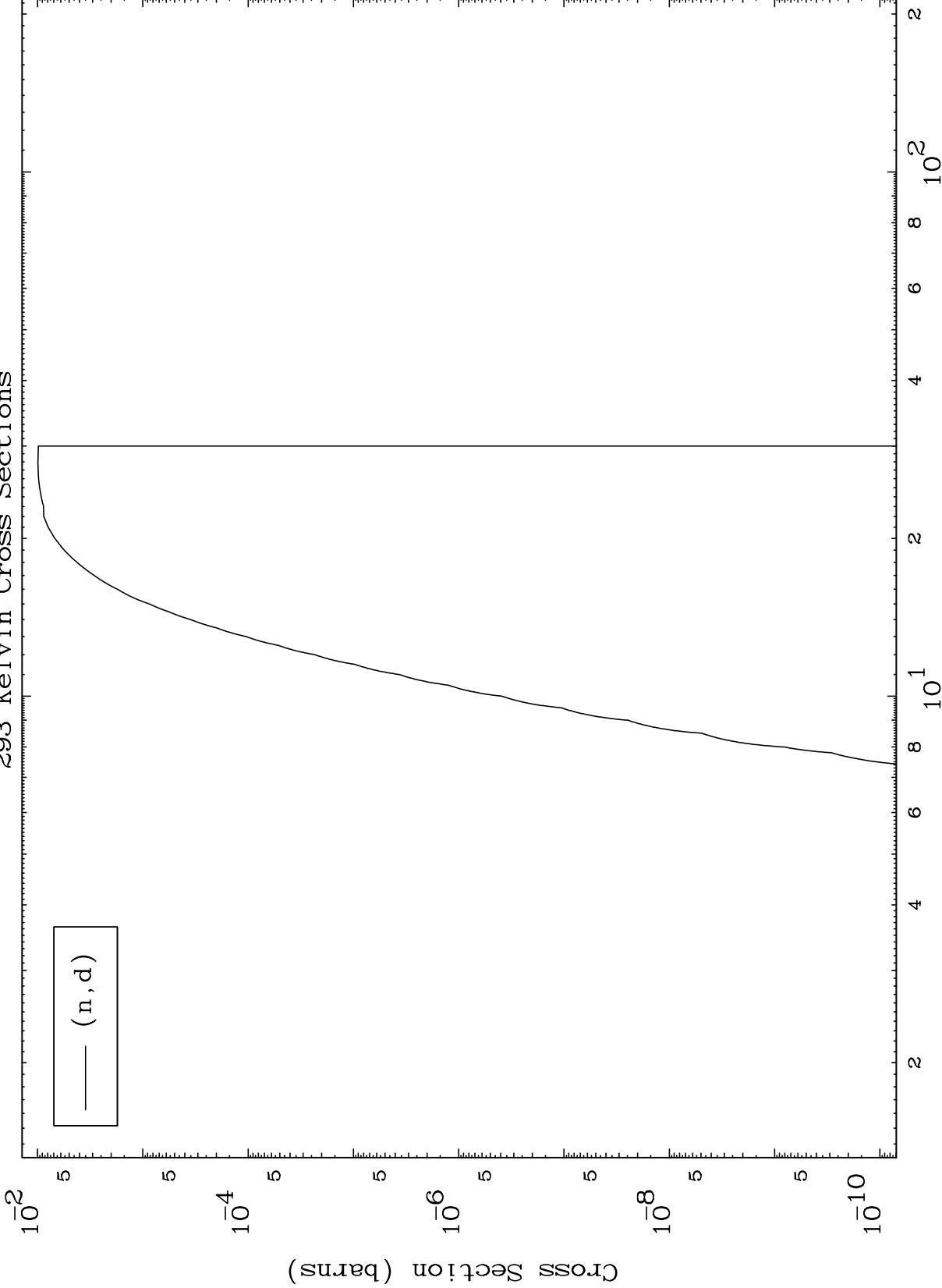
101-Md-256



MAT 9957

(n,d) Levels
293 Kelvin Cross Sections

101-Md-256

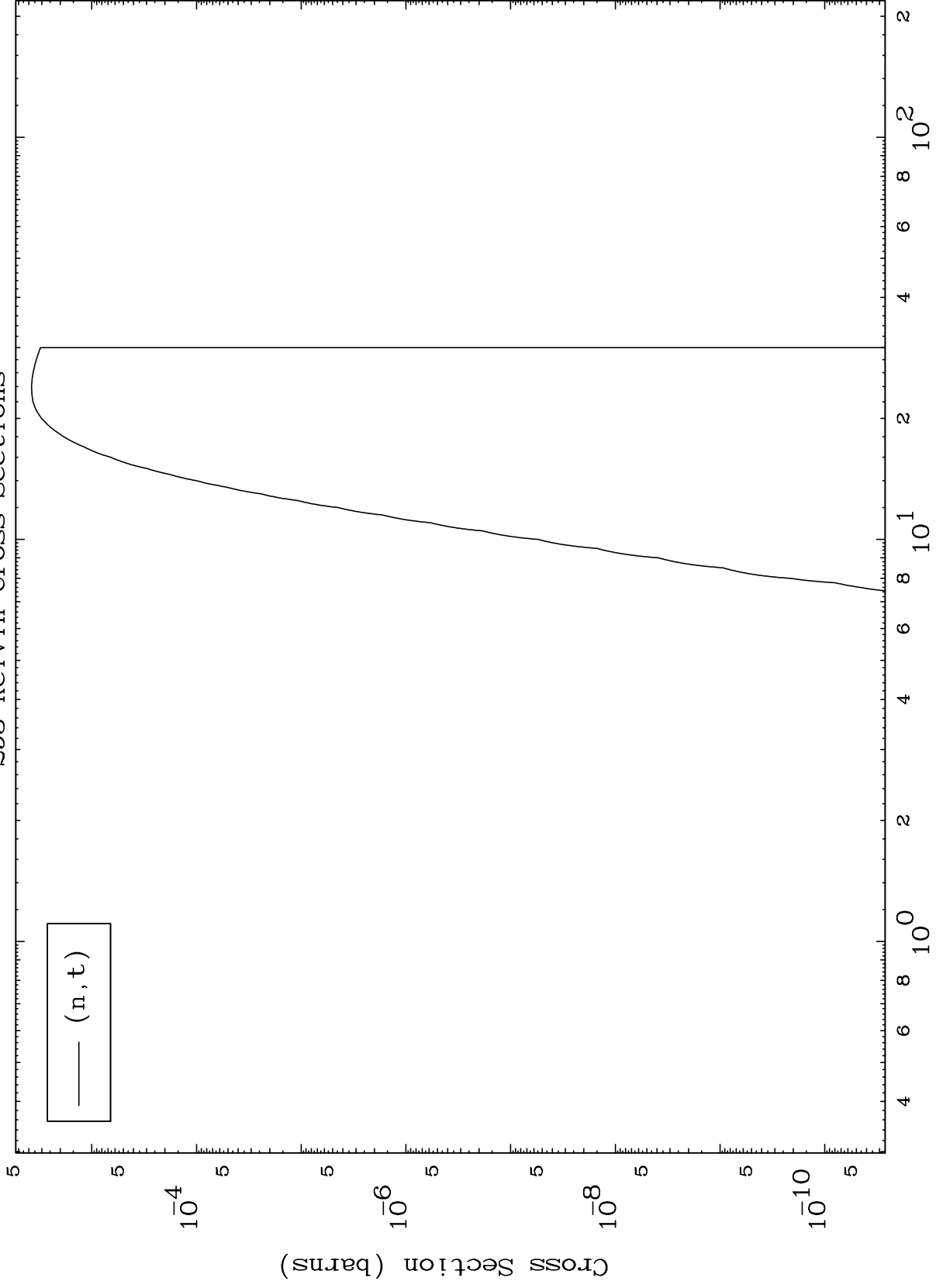


(n,d)

MAT 9957

(n,t) Levels
293 Kelvin Cross Sections

101-Md-256



15

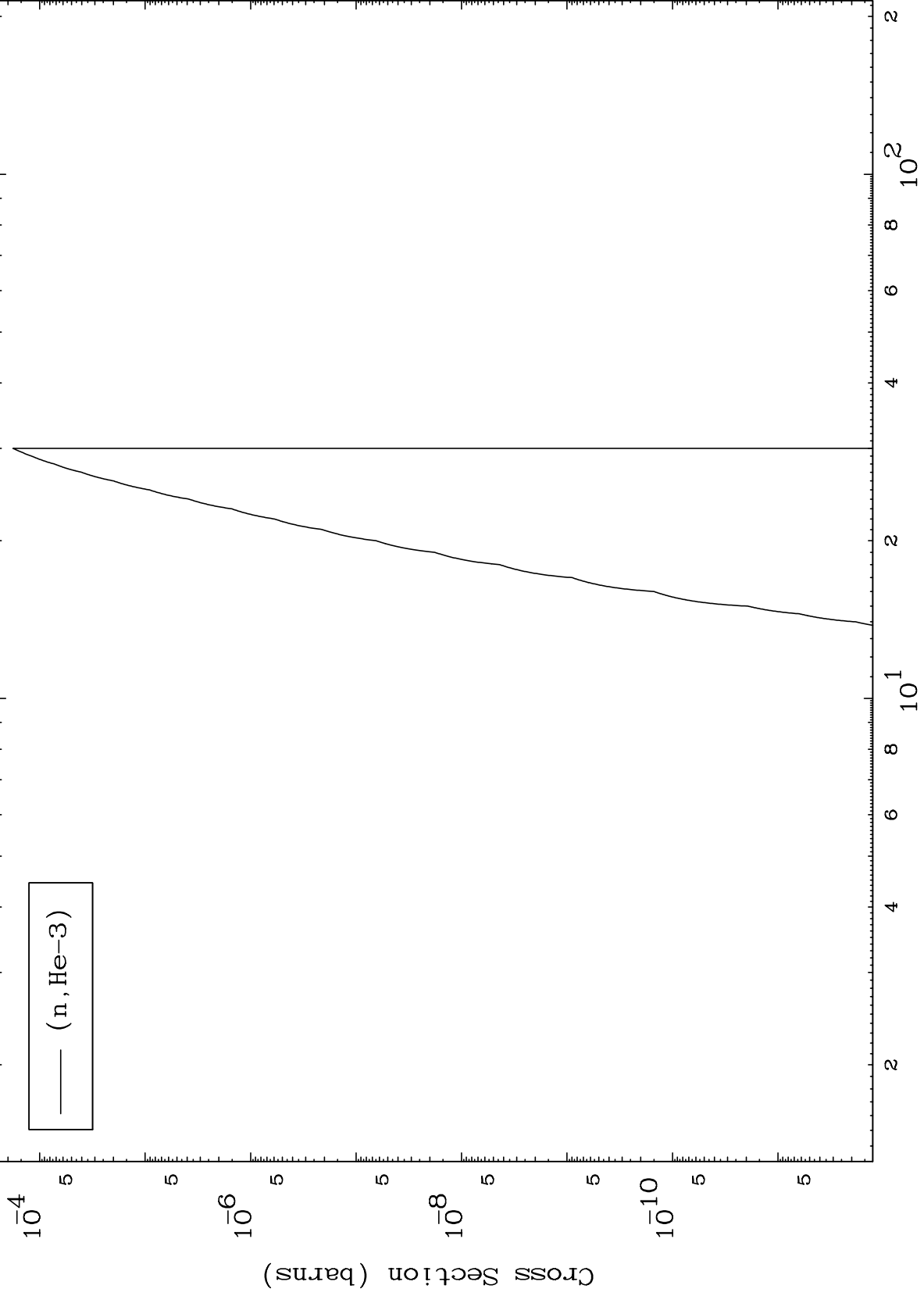
Incident Energy (MeV)

101-Md-256

MAT 9957

(n,He3) Levels
293 Kelvin Cross Sections

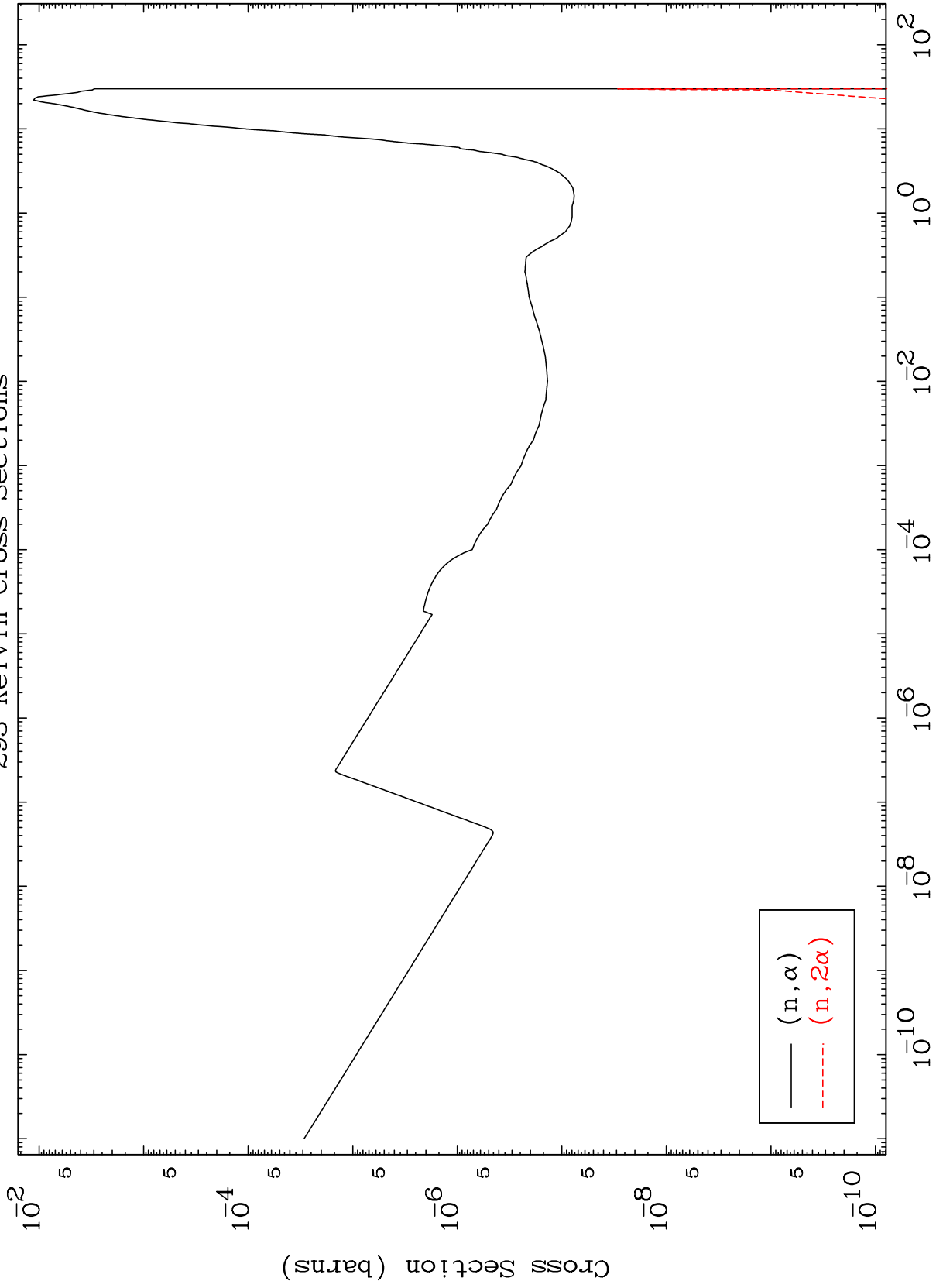
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MAT 9957

(n, α) Levels
293 Kelvin Cross Sections

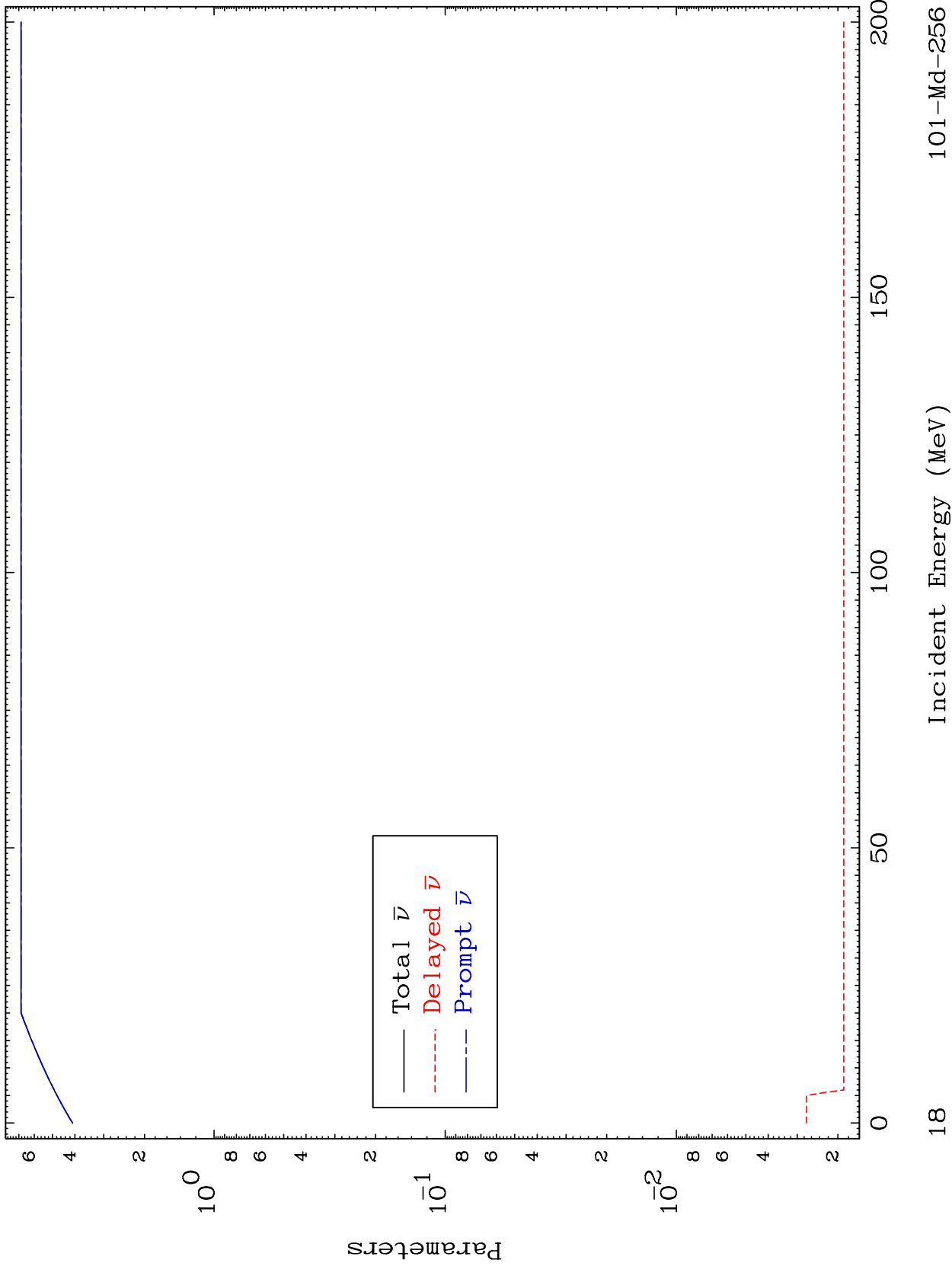
101-Md-256



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Energy Release
Parameters

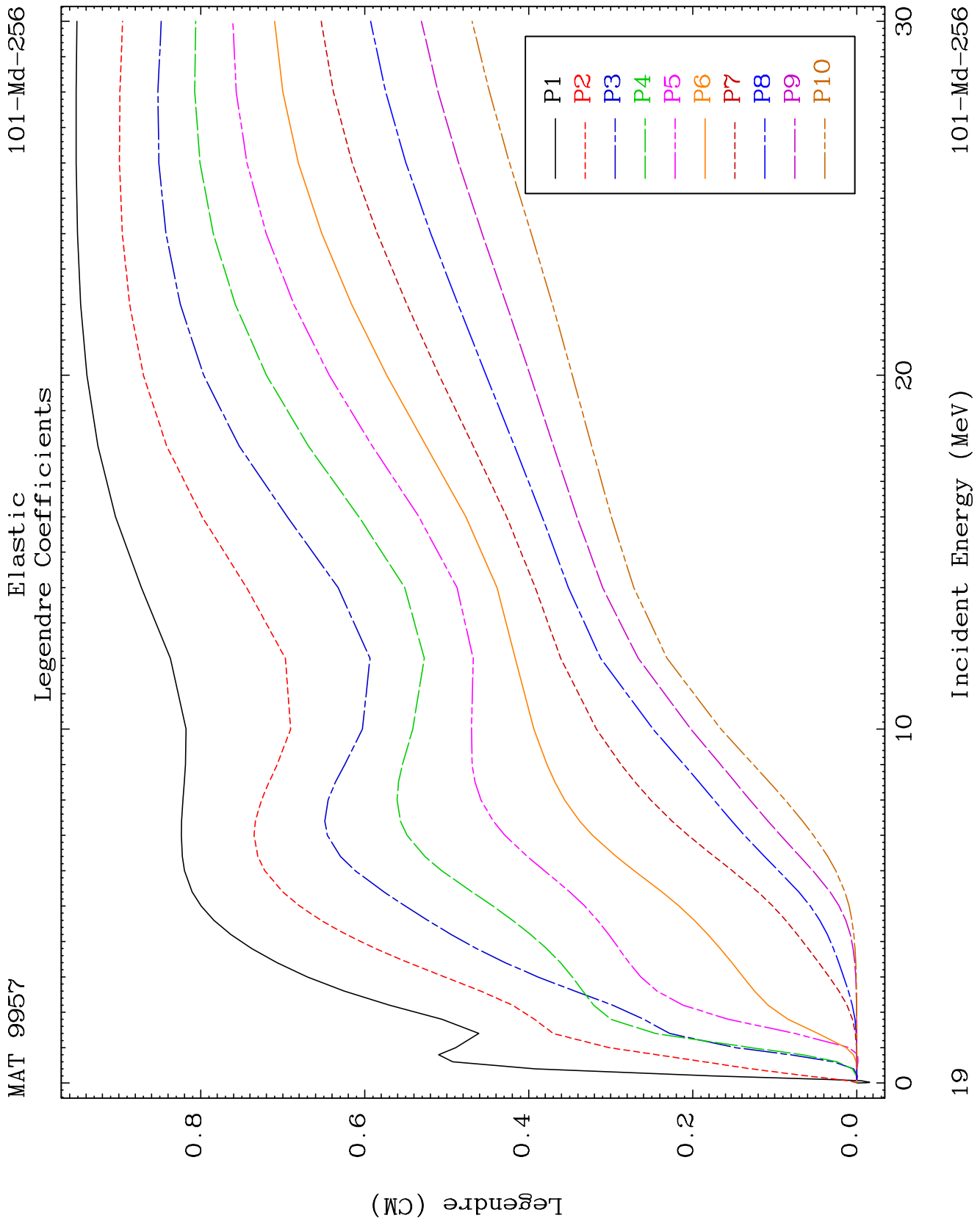
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18

Incident Energy (MeV)

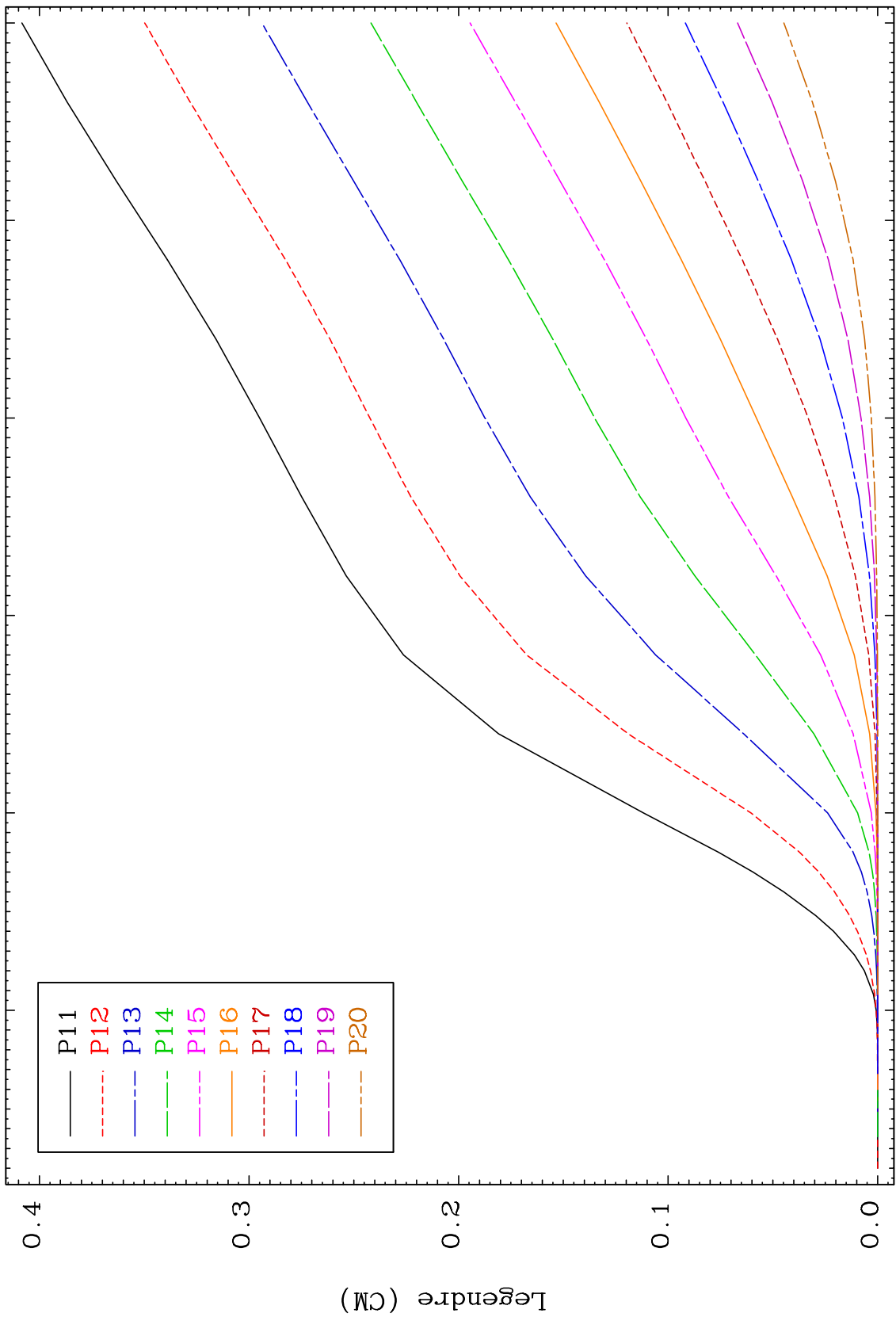
101-Md-256



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Elastic Legendre Coefficients

101-Md-256

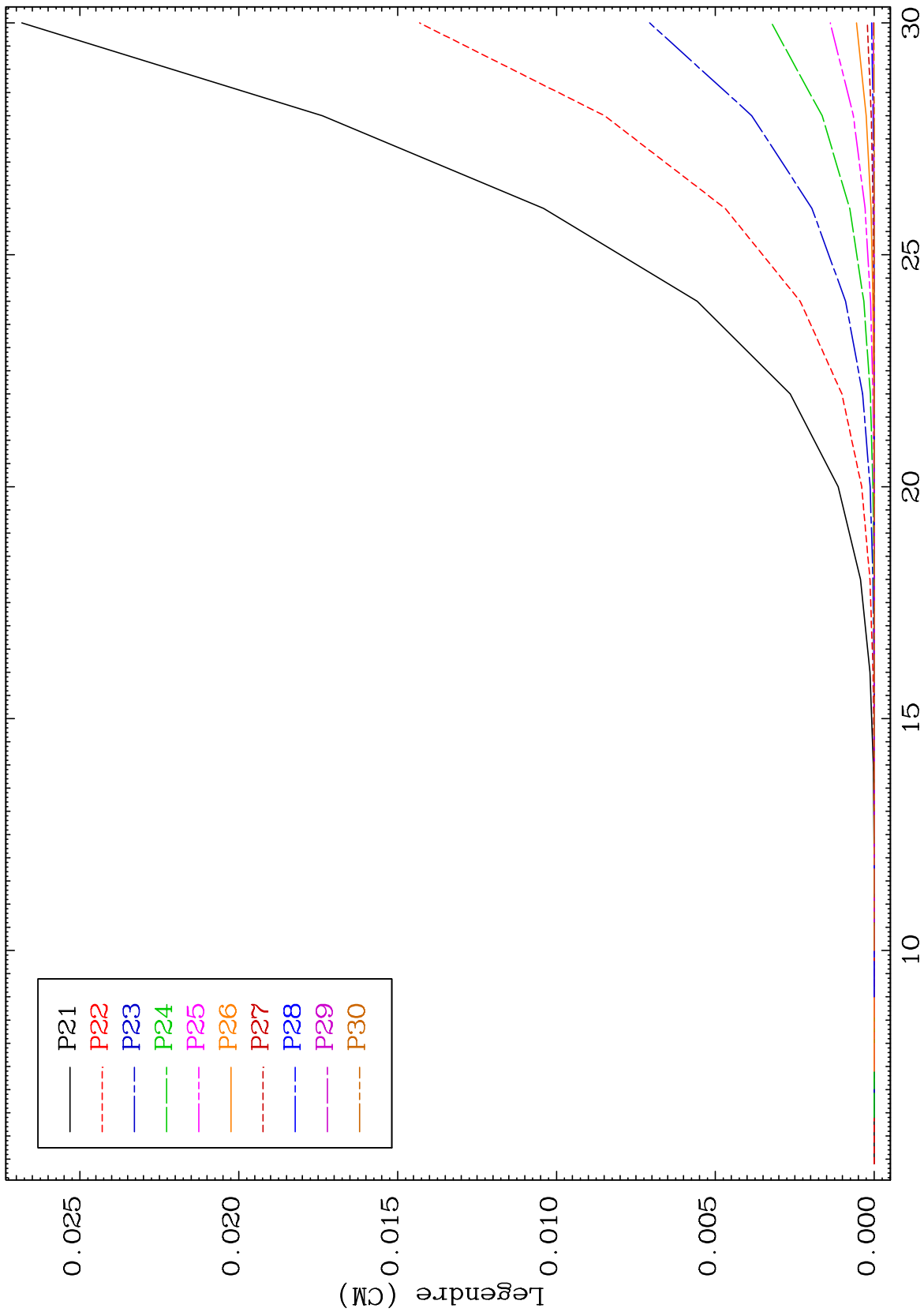


20

Incident Energy (MeV)

101-Md-256

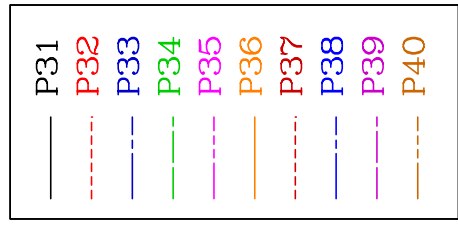
Elastic Legendre Coefficients



MAT 99577

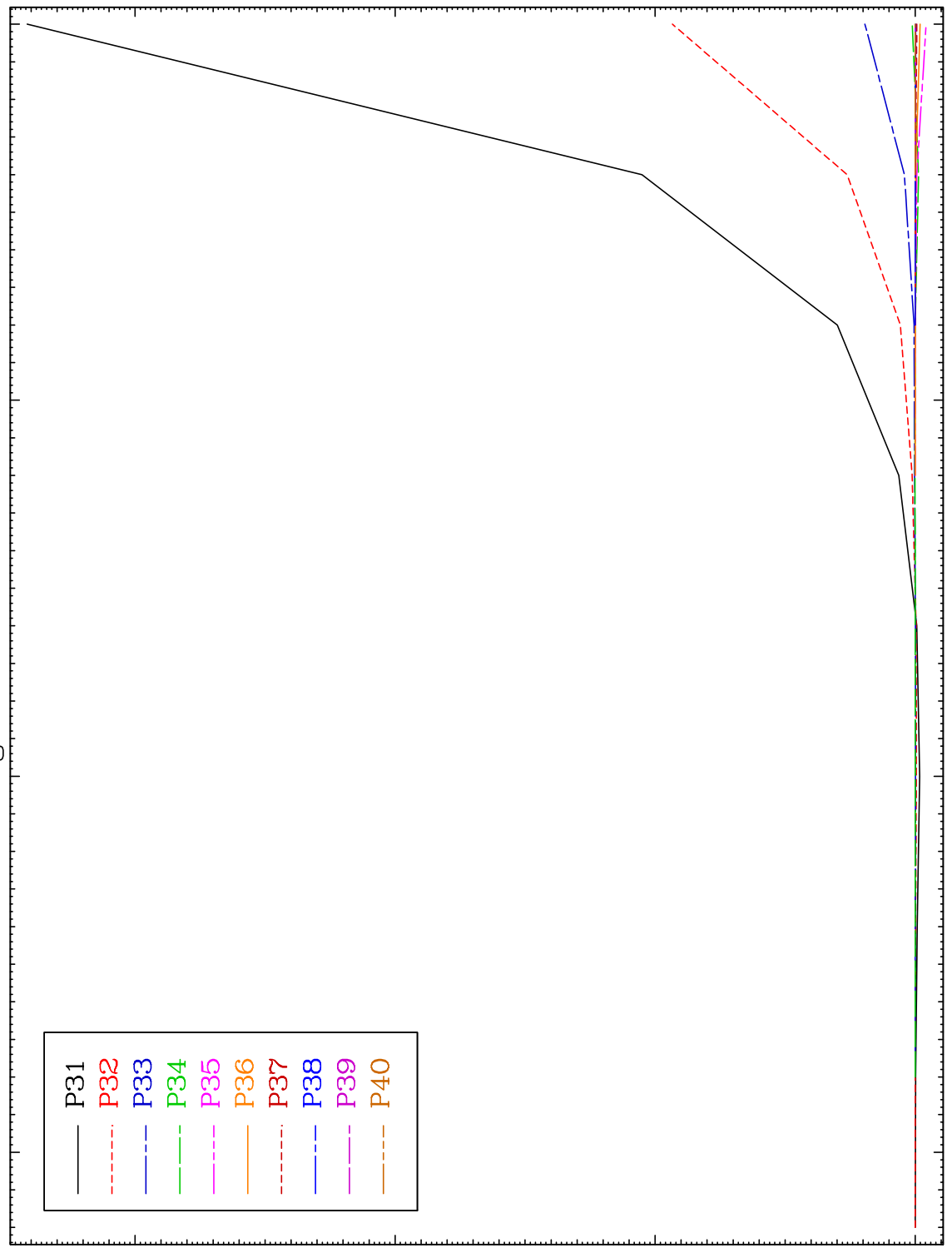
Elastic Legendre Coefficients

101-Md-256



$\times 10^{-6}$

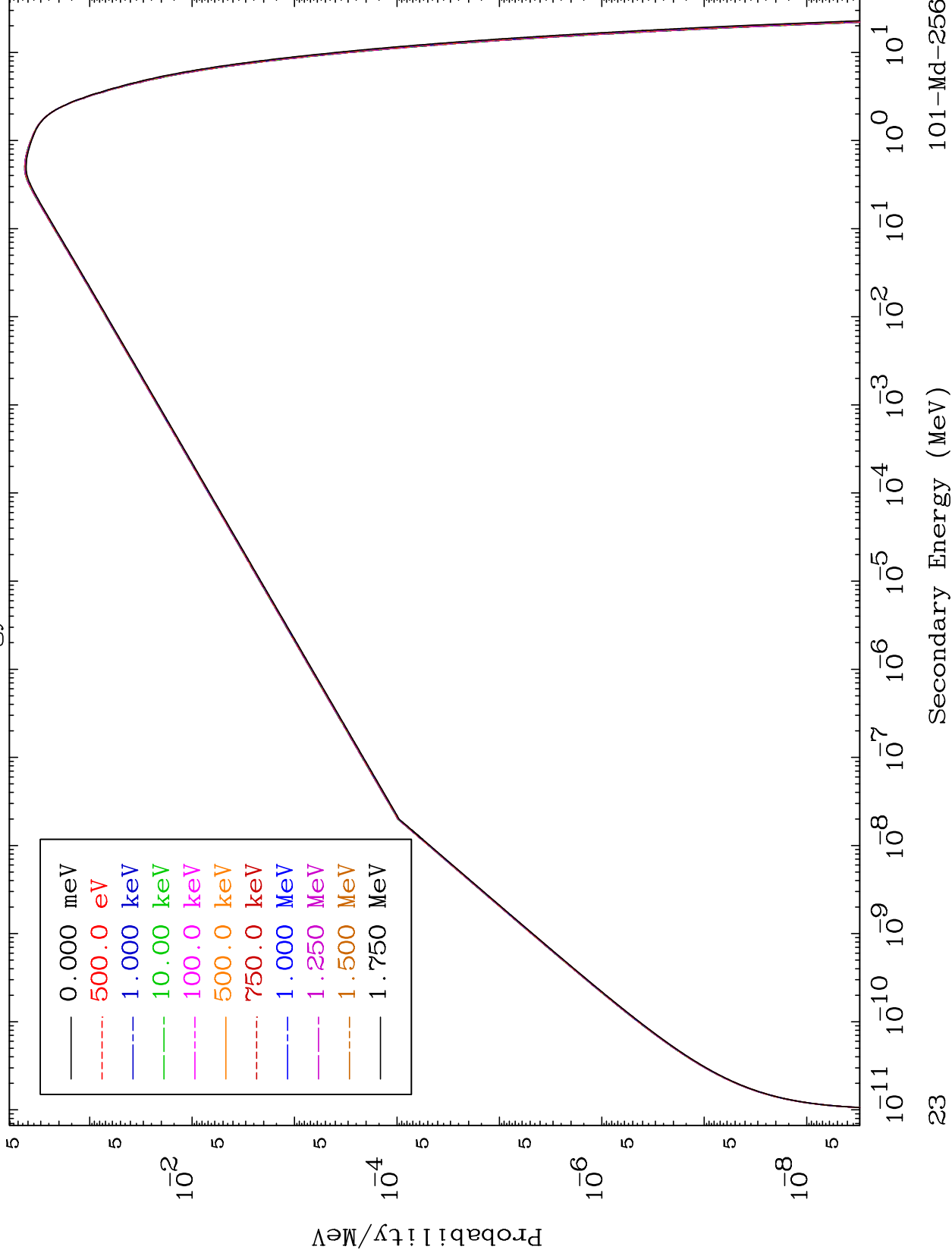
Legendre (CM)



22

Incident Energy (MeV)

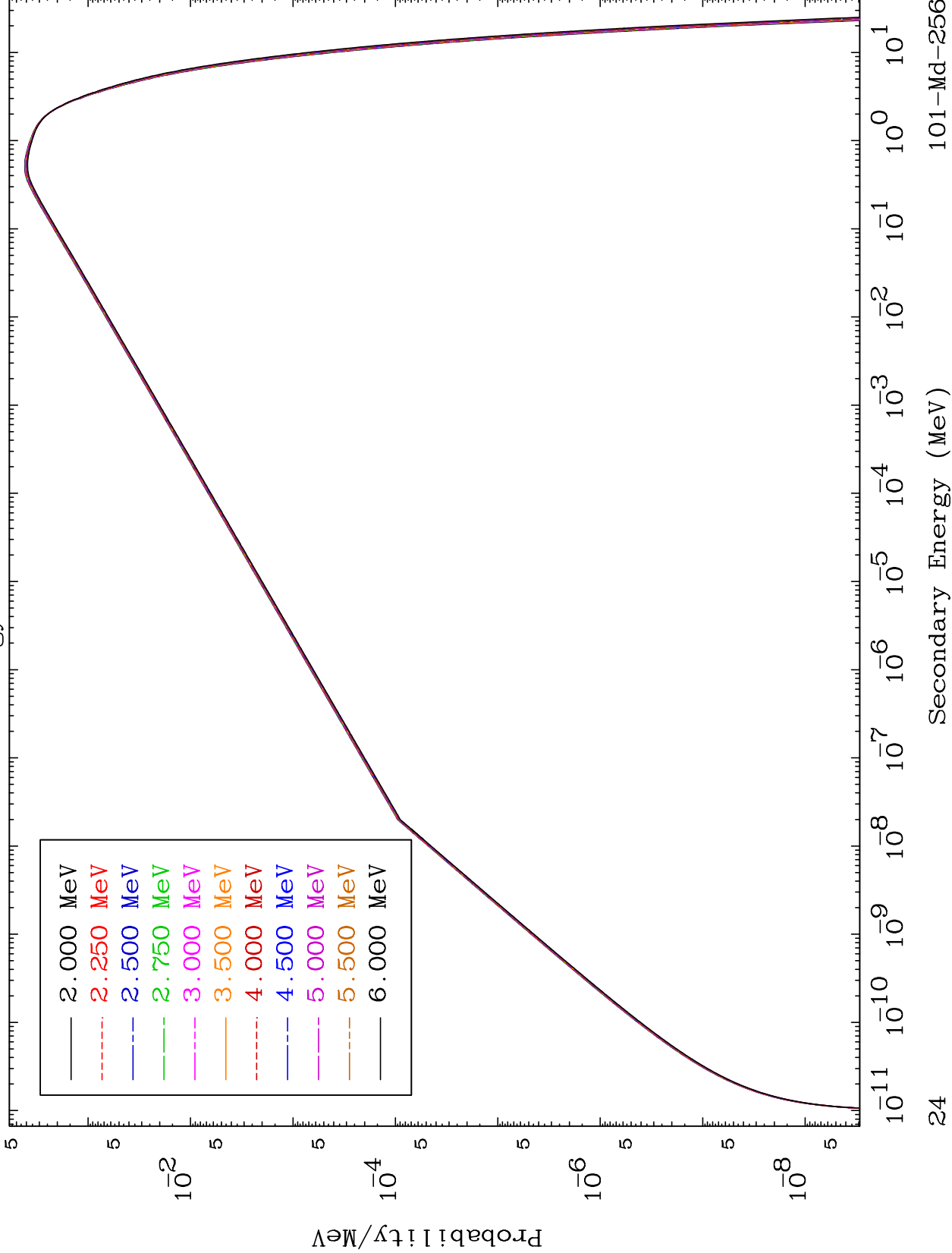
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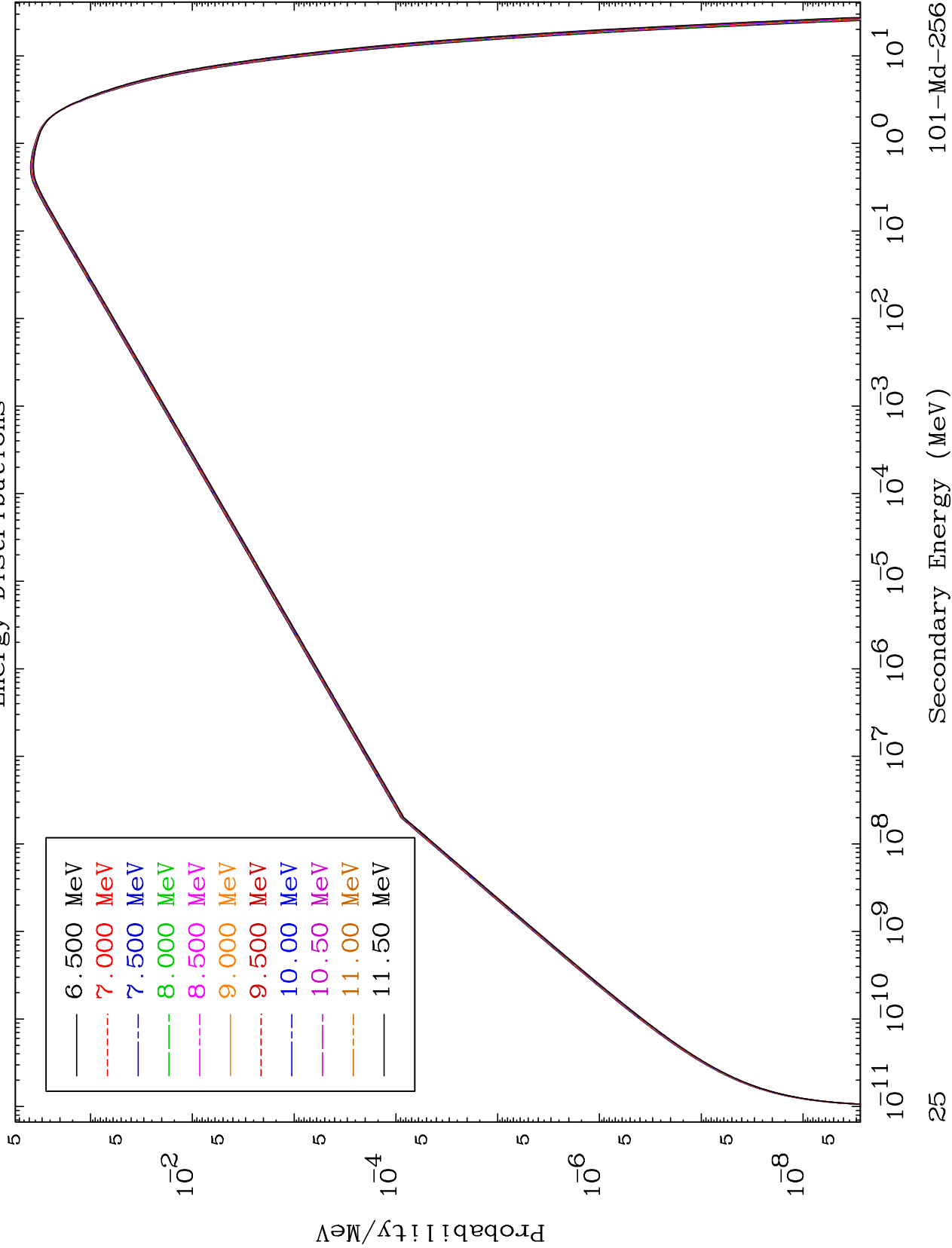
MAT 9957

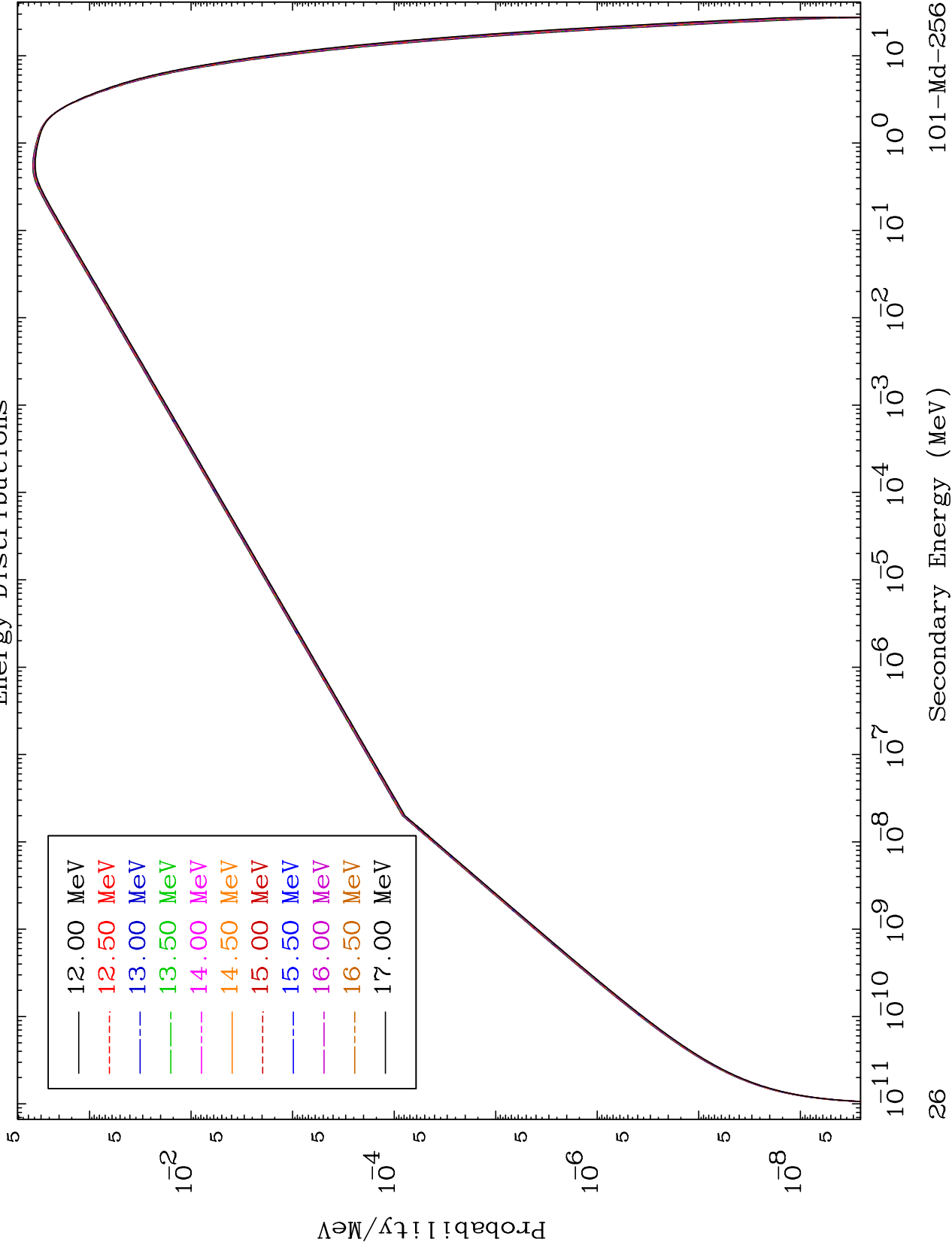
Fission
Energy Distributions

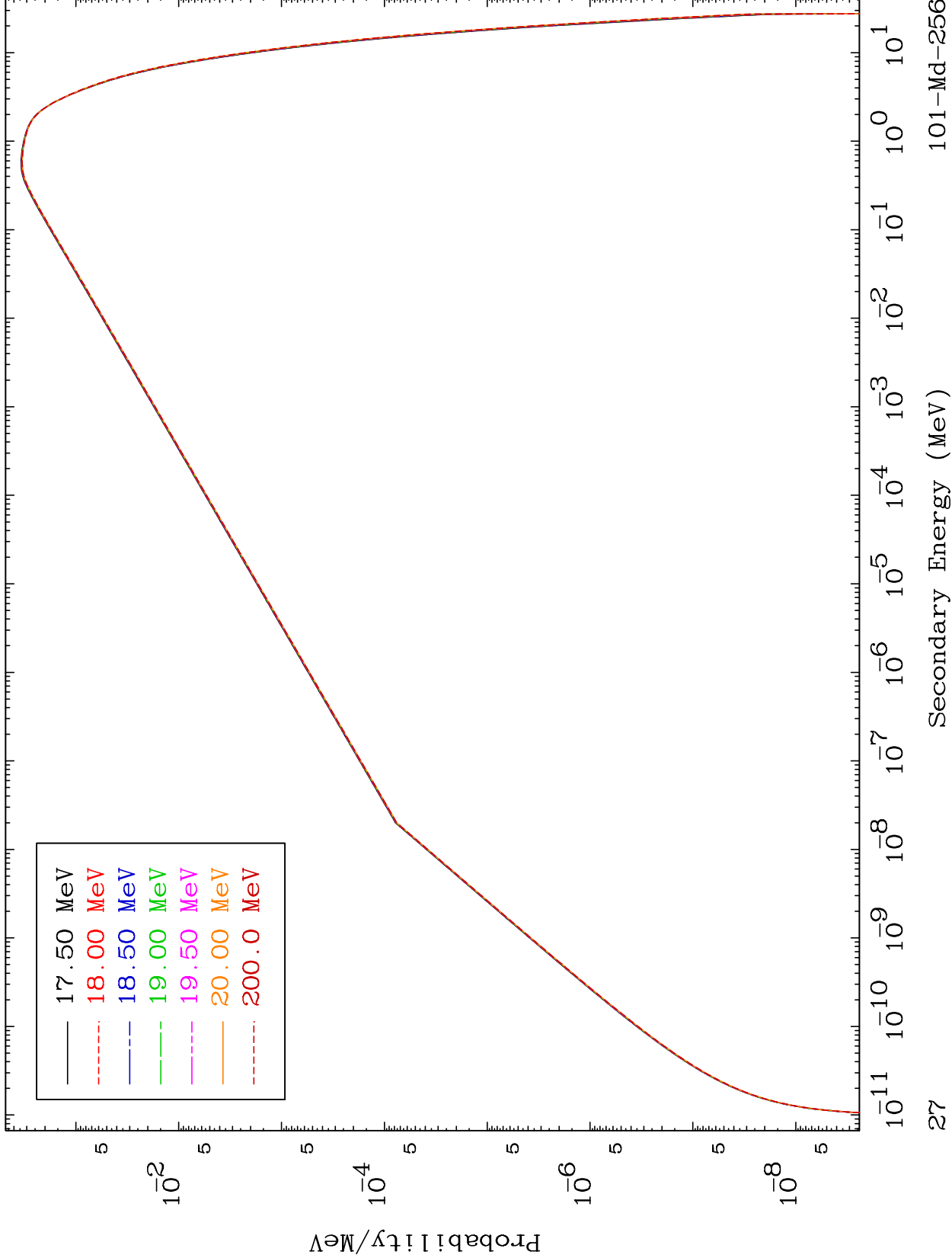
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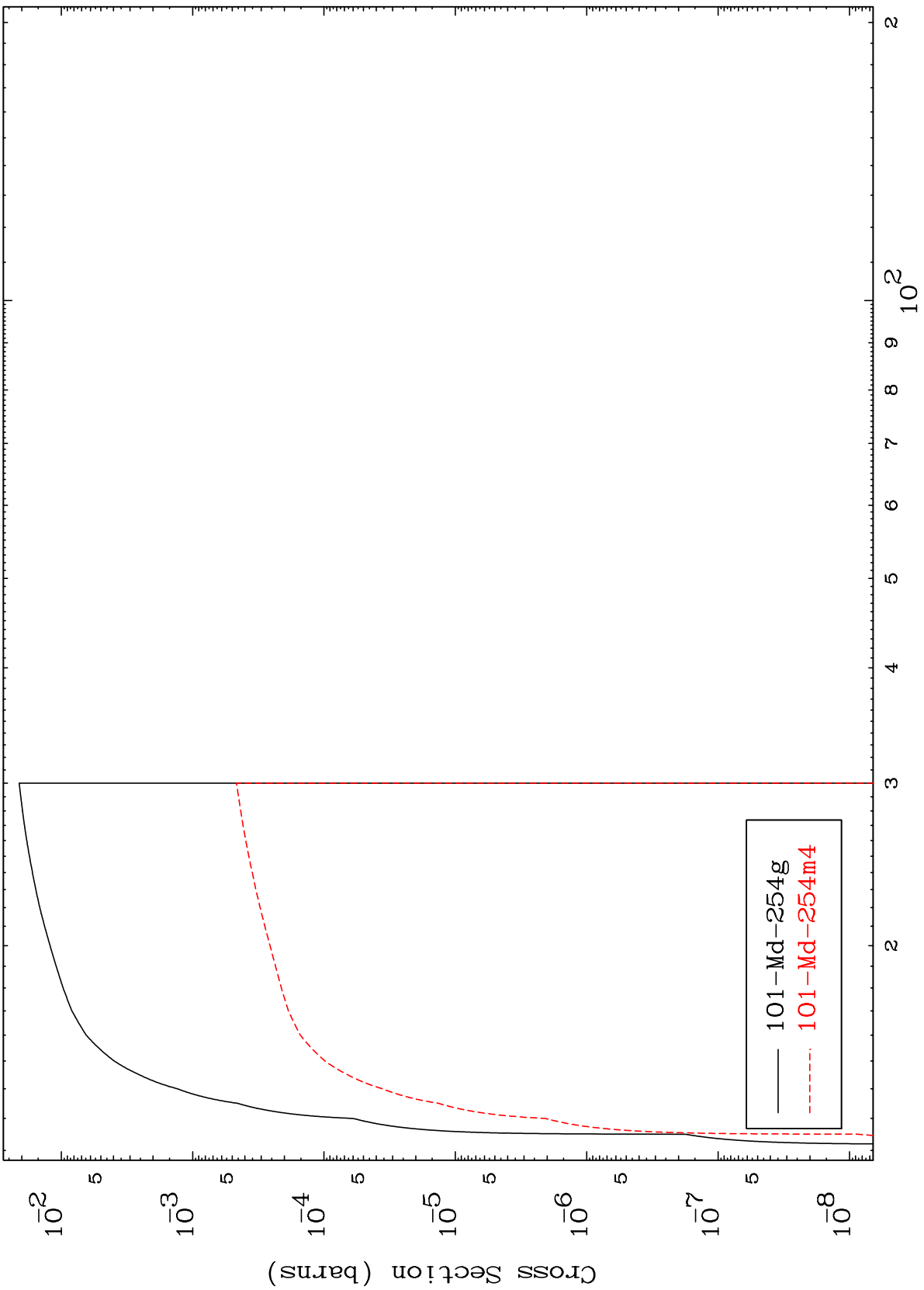
Fission Energy Distributions







(n,3n)
Radionuclide Production Cross Section

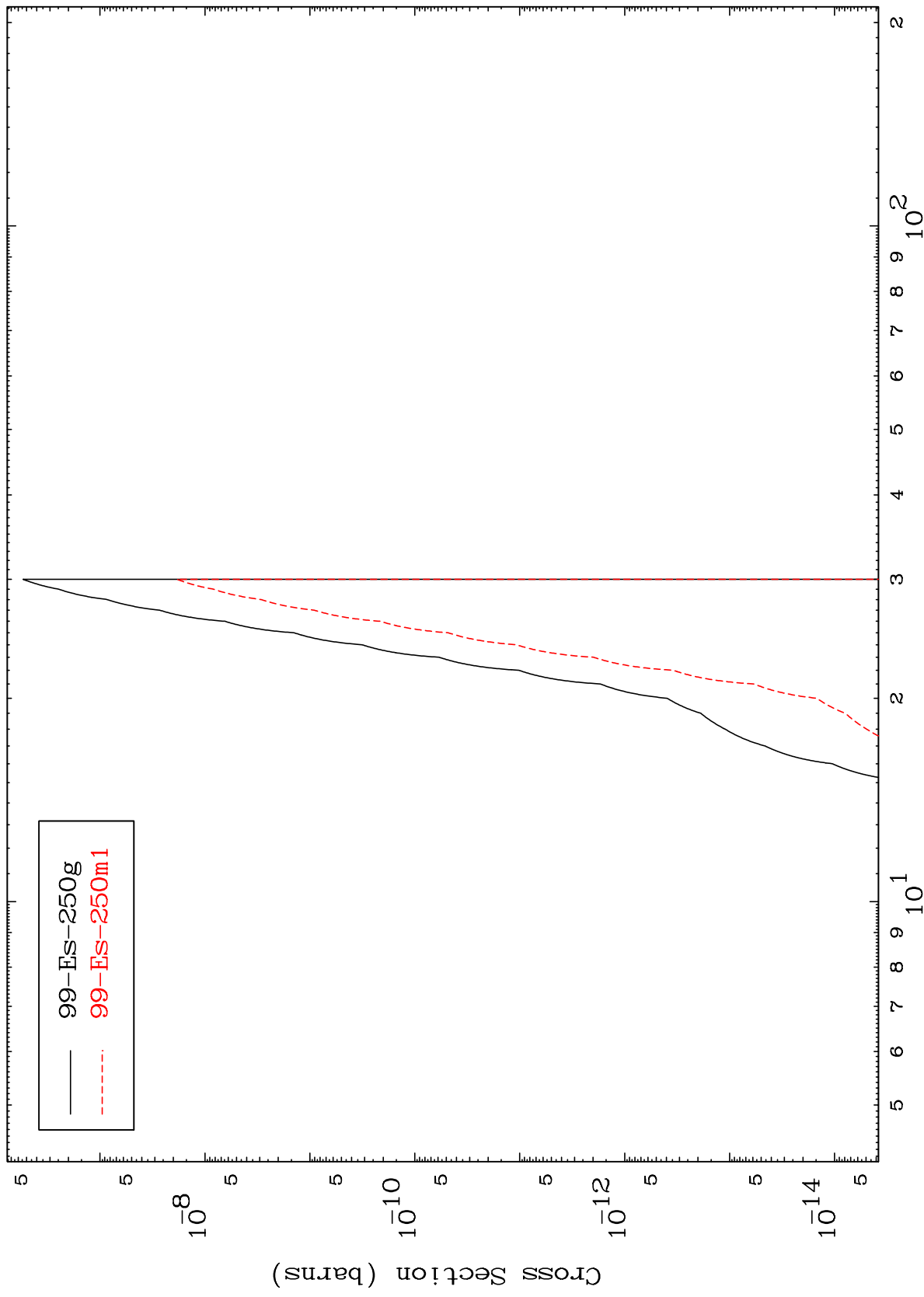


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(n,3n) α

101-Md-256

Radionuclide Production Cross Section



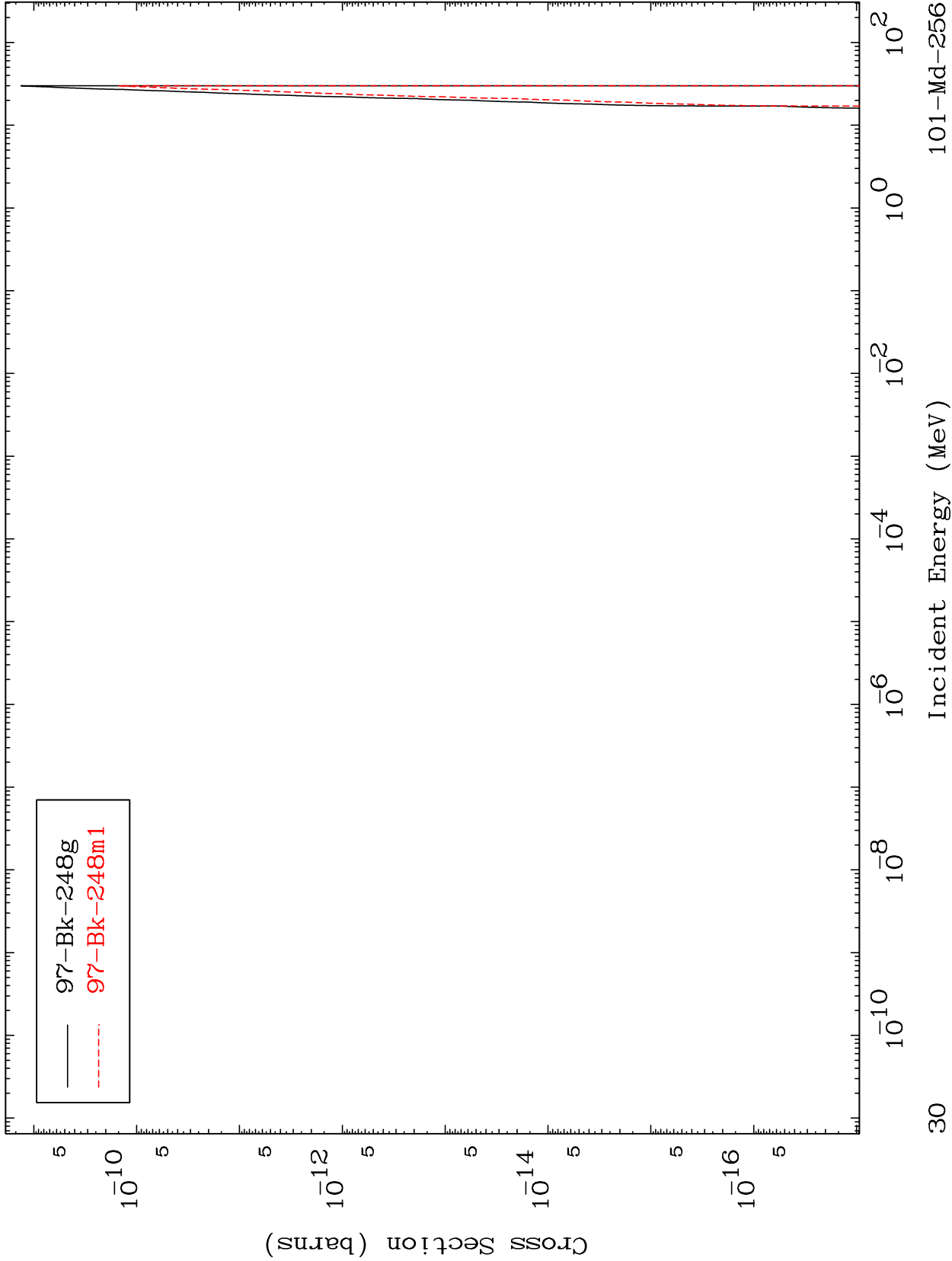
99-Es-250g
99-Es-250m1

MAT 9957

(n,n') 2 α

101-Md-256

Radionuclide Production Cross Section

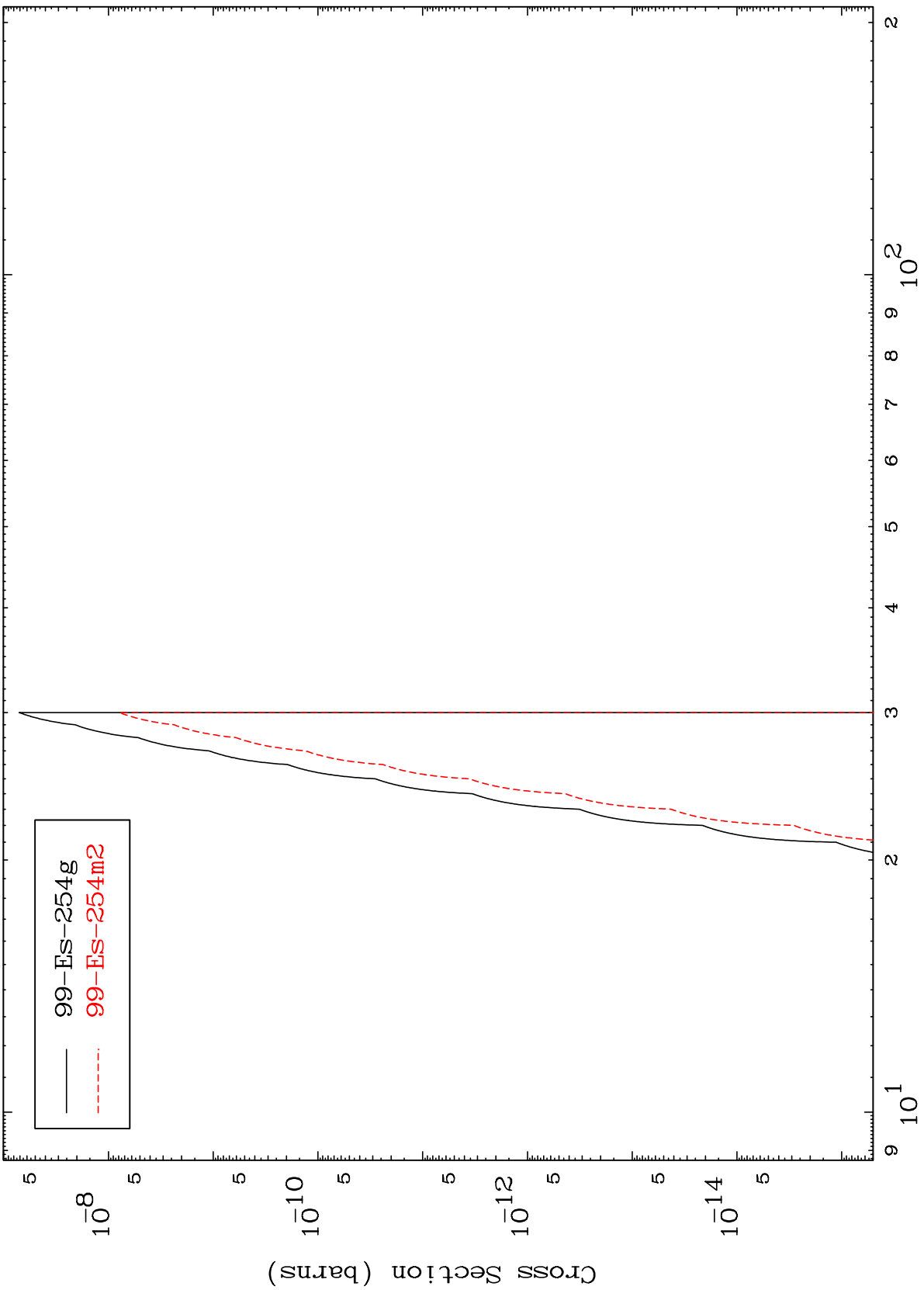


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(n,2n) p

101-Md-256

Radionuclide Production Cross Section



31

Incident Energy (MeV)

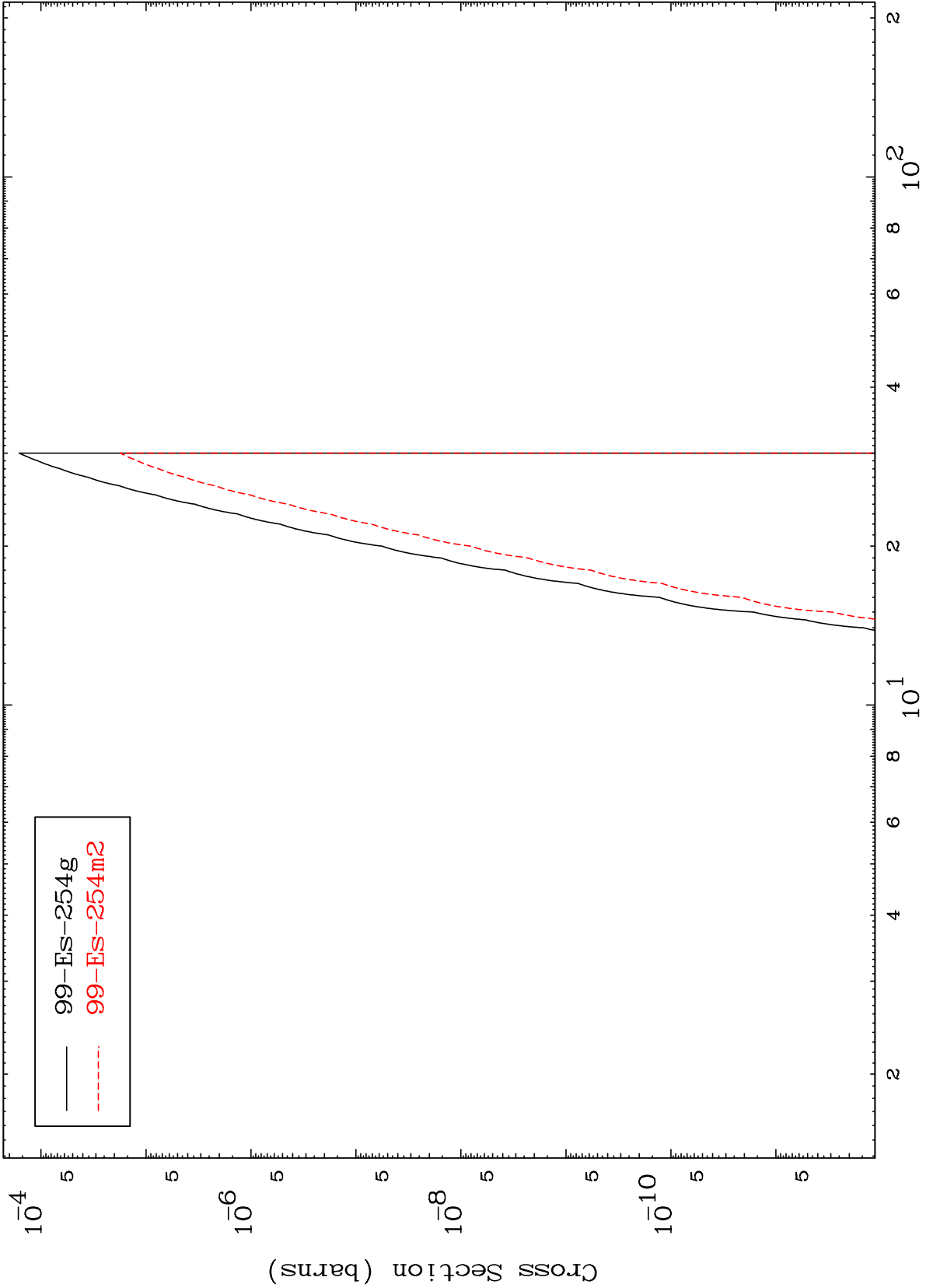
101-Md-256

MAT 9957

(n,He-3)

101-Md-256

Radionuclide Production Cross Section



— 99-Es-254g
- - - 99-Es-254m2