

Program EVALPLOT  
(Version 2021-1)

by

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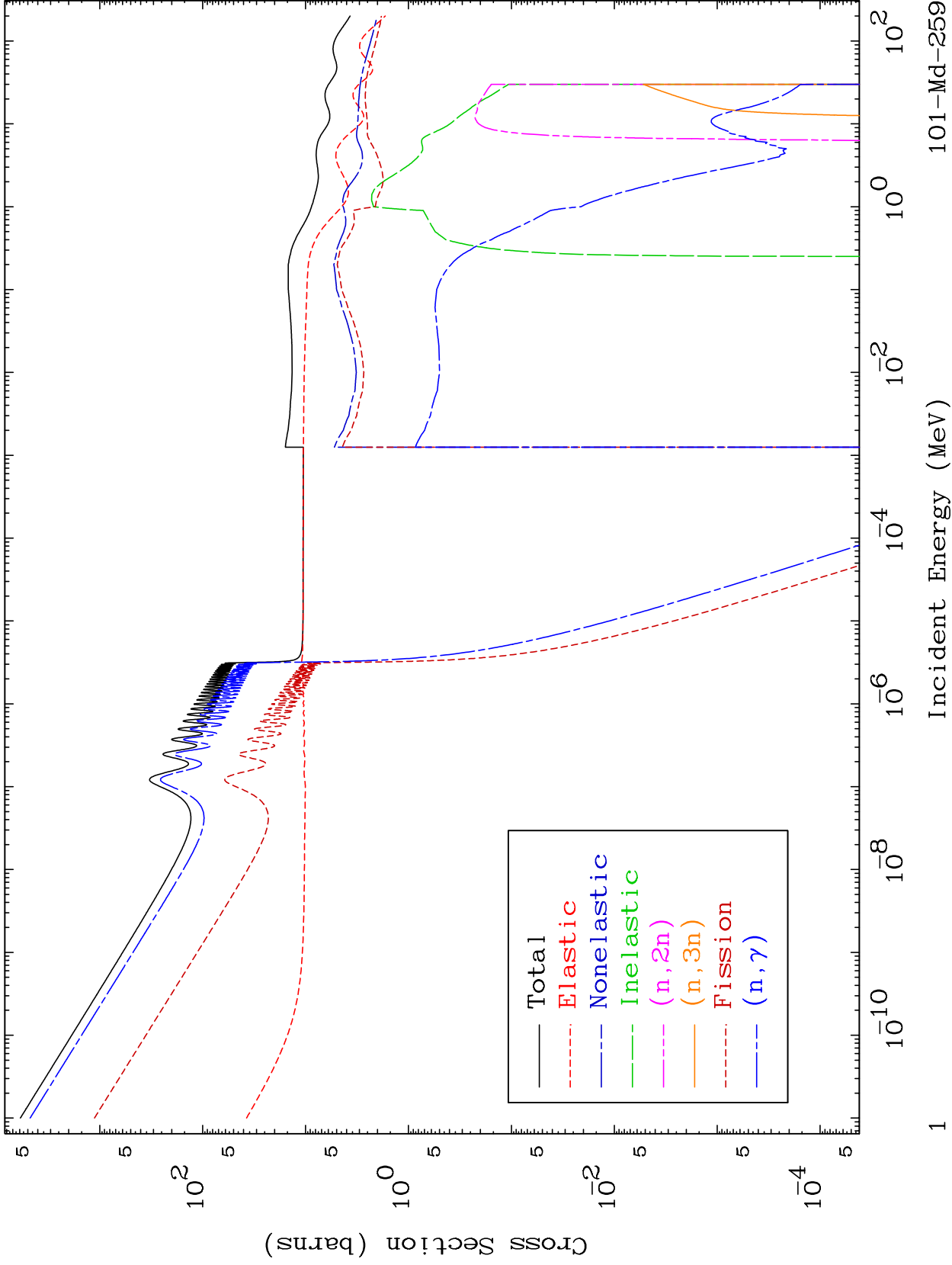
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Press Mouse Button to Start

MAT 9960

Neutron Major  
293 Kelvin Cross Sections

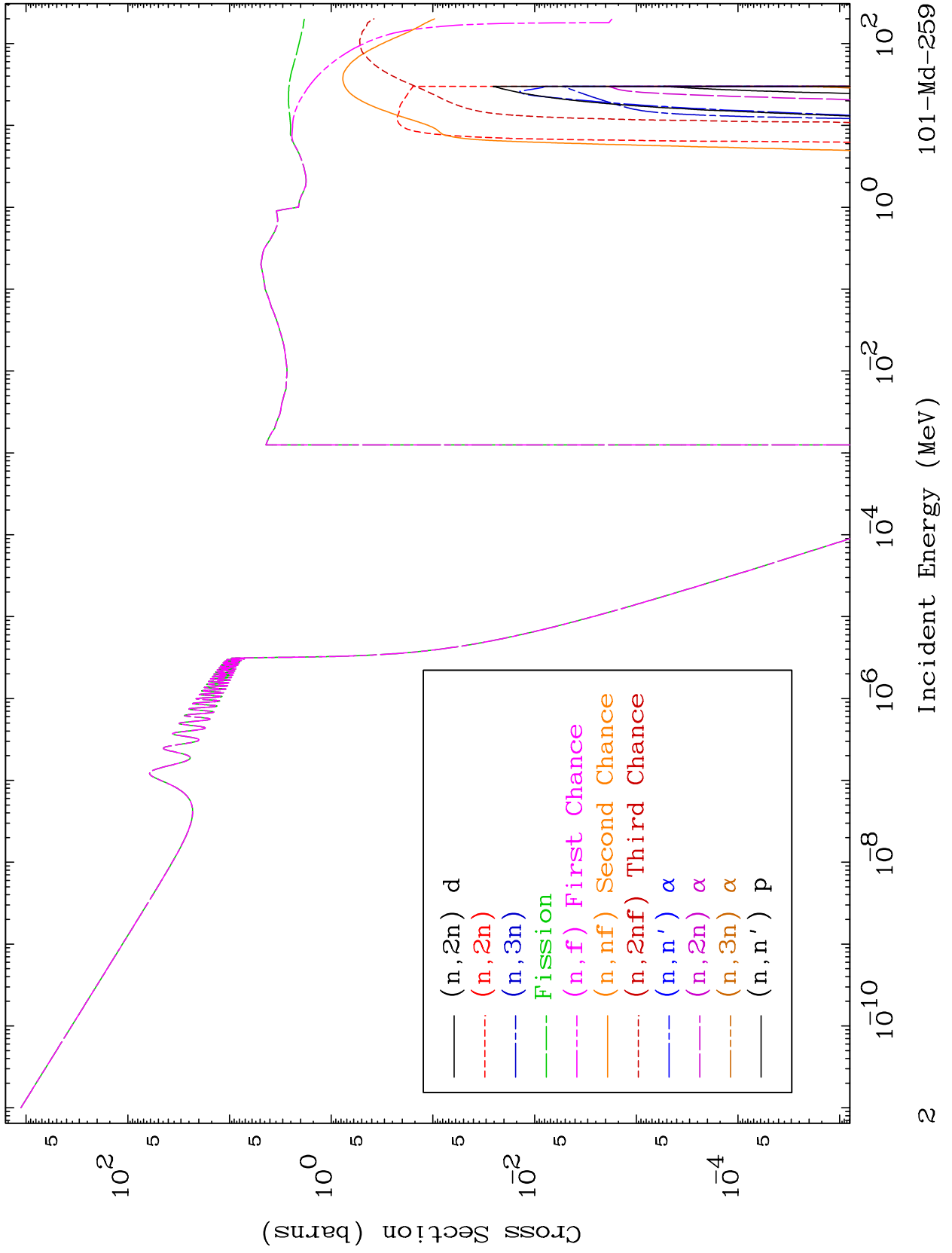
101-Md-259



MAT 9960

Neutron Absorption  
293 Kelvin Cross Sections

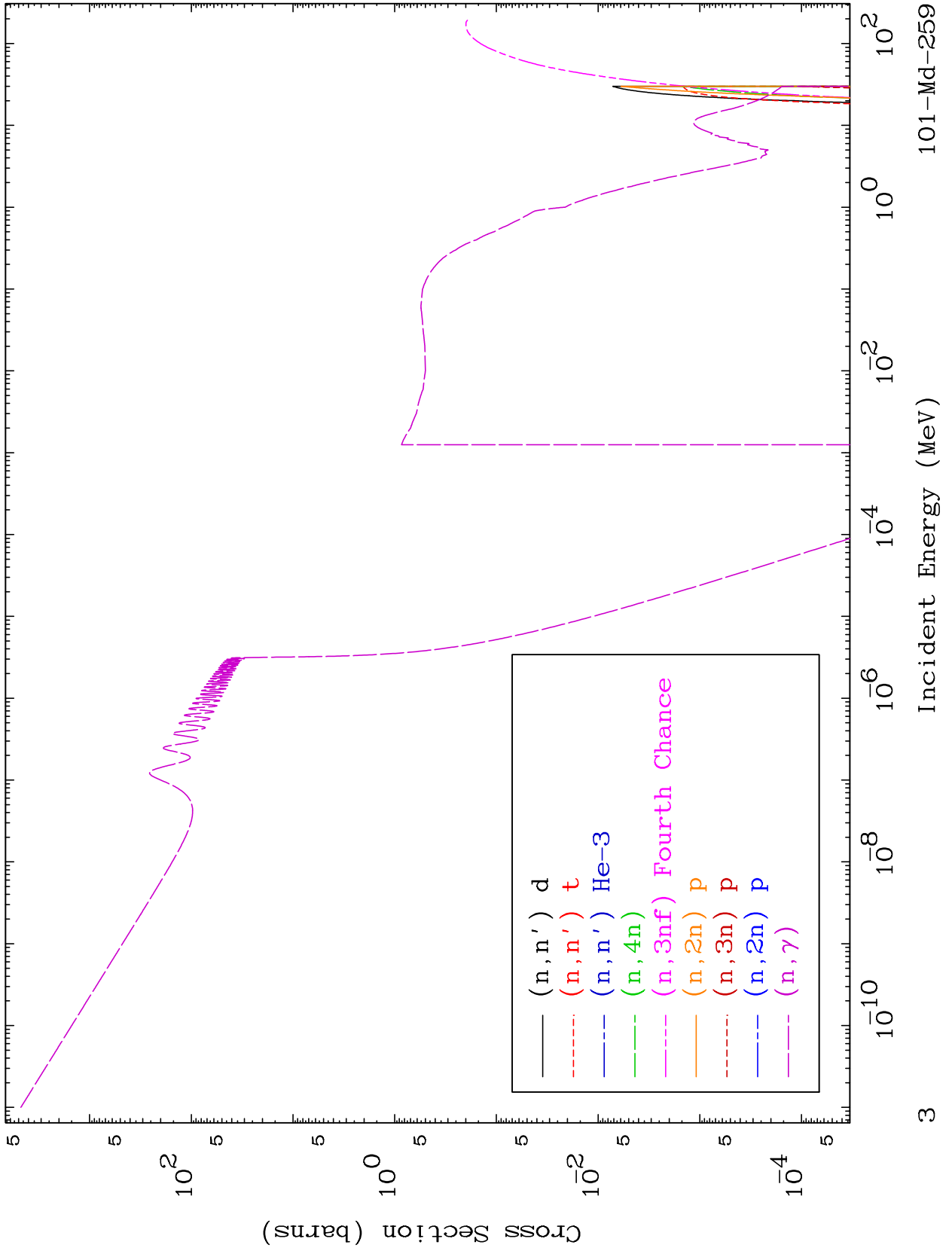
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MAT 9960

Neutron Absorption  
293 Kelvin Cross Sections

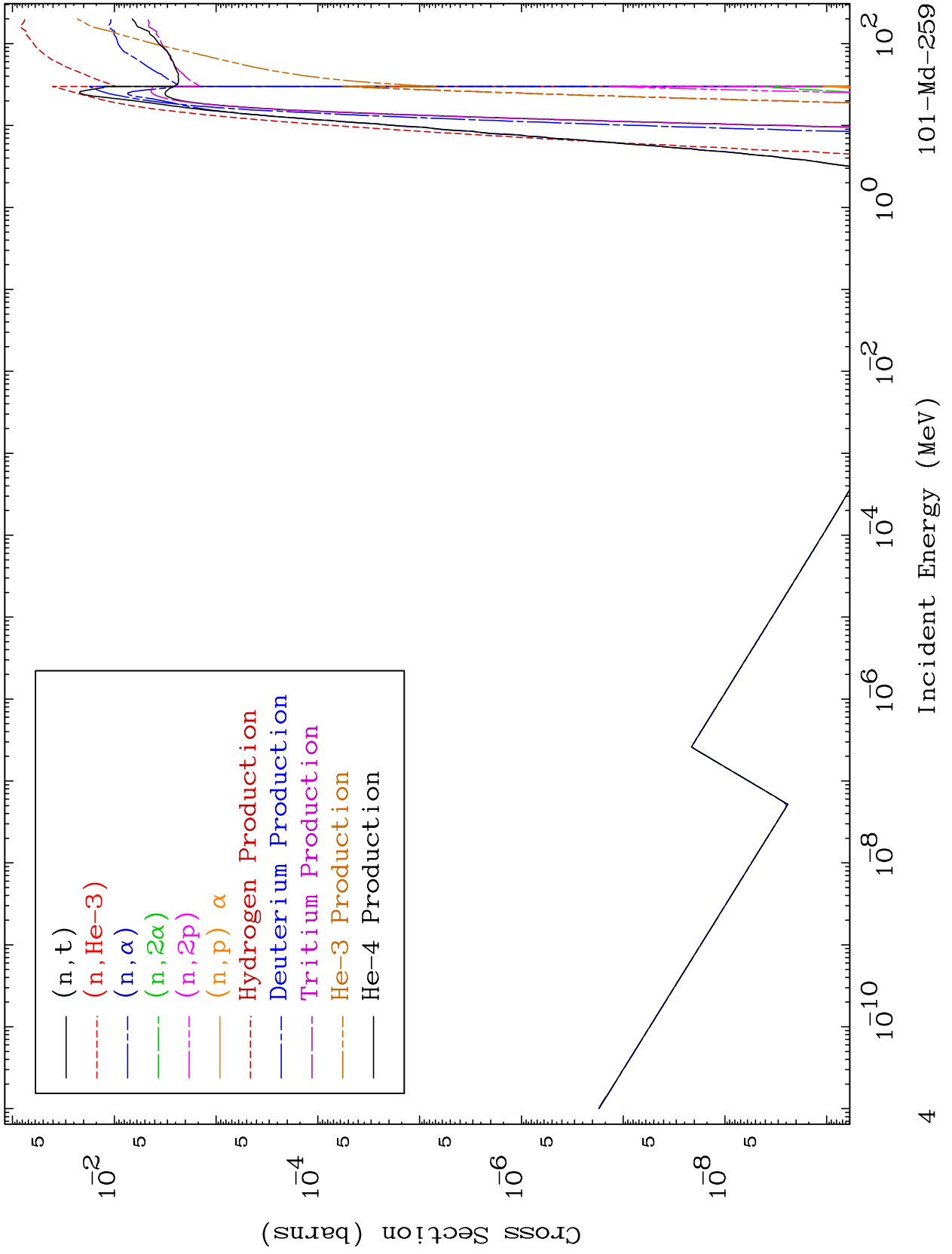
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Neutron Absorption  
293 Kelvin Cross Sections

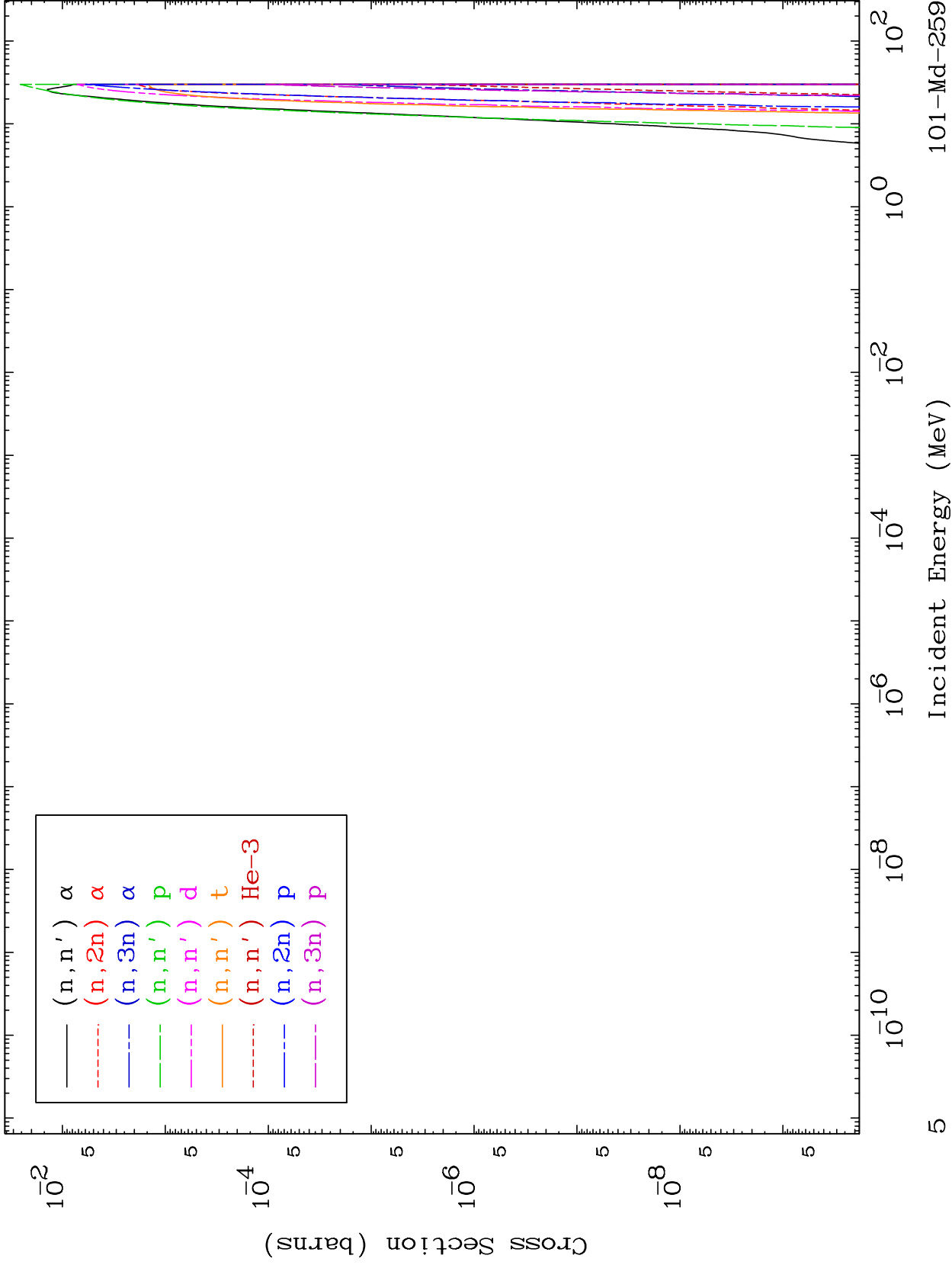
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Charged Particle  
293 Kelvin Cross Sections

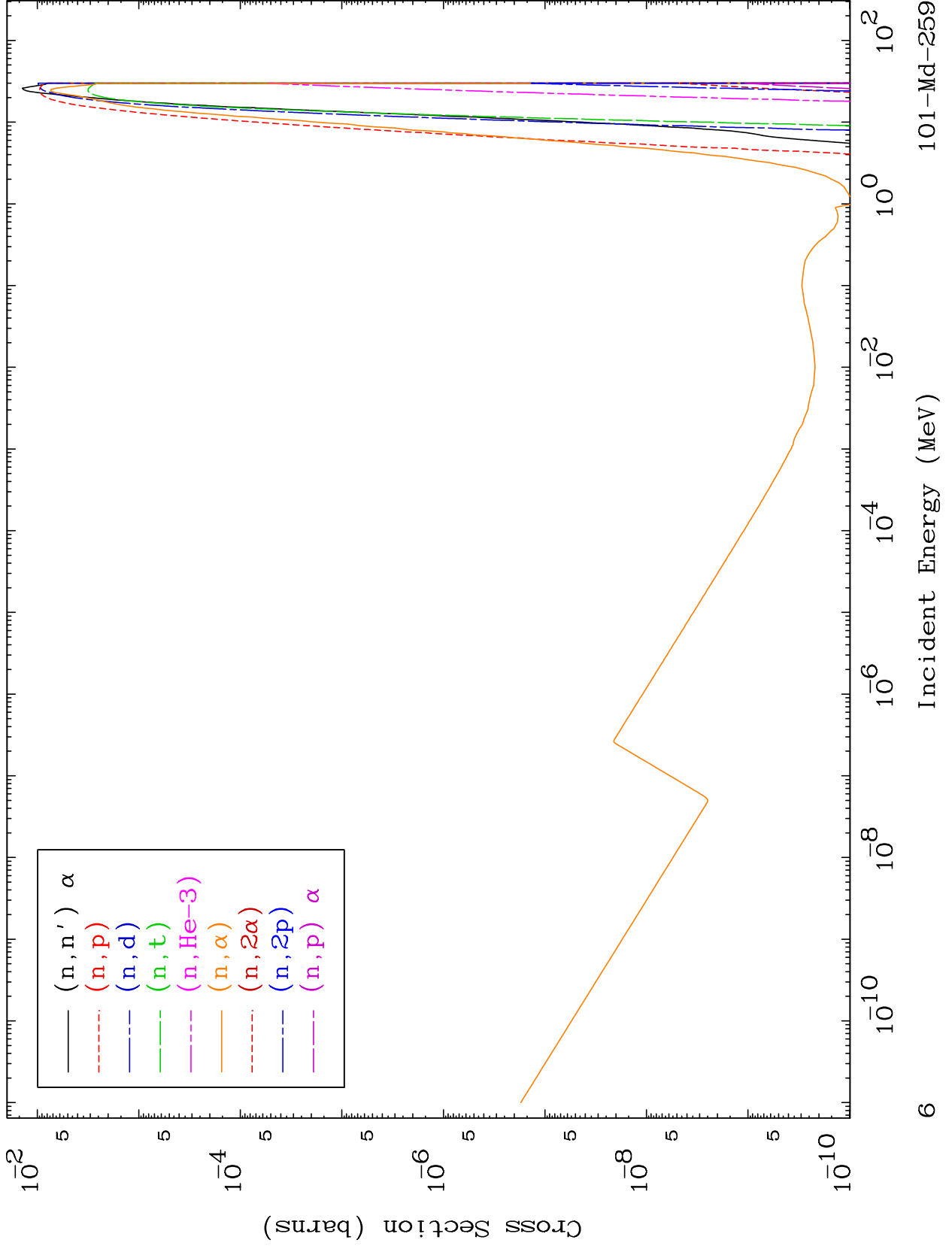
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Charged Particle  
293 Kelvin Cross Sections

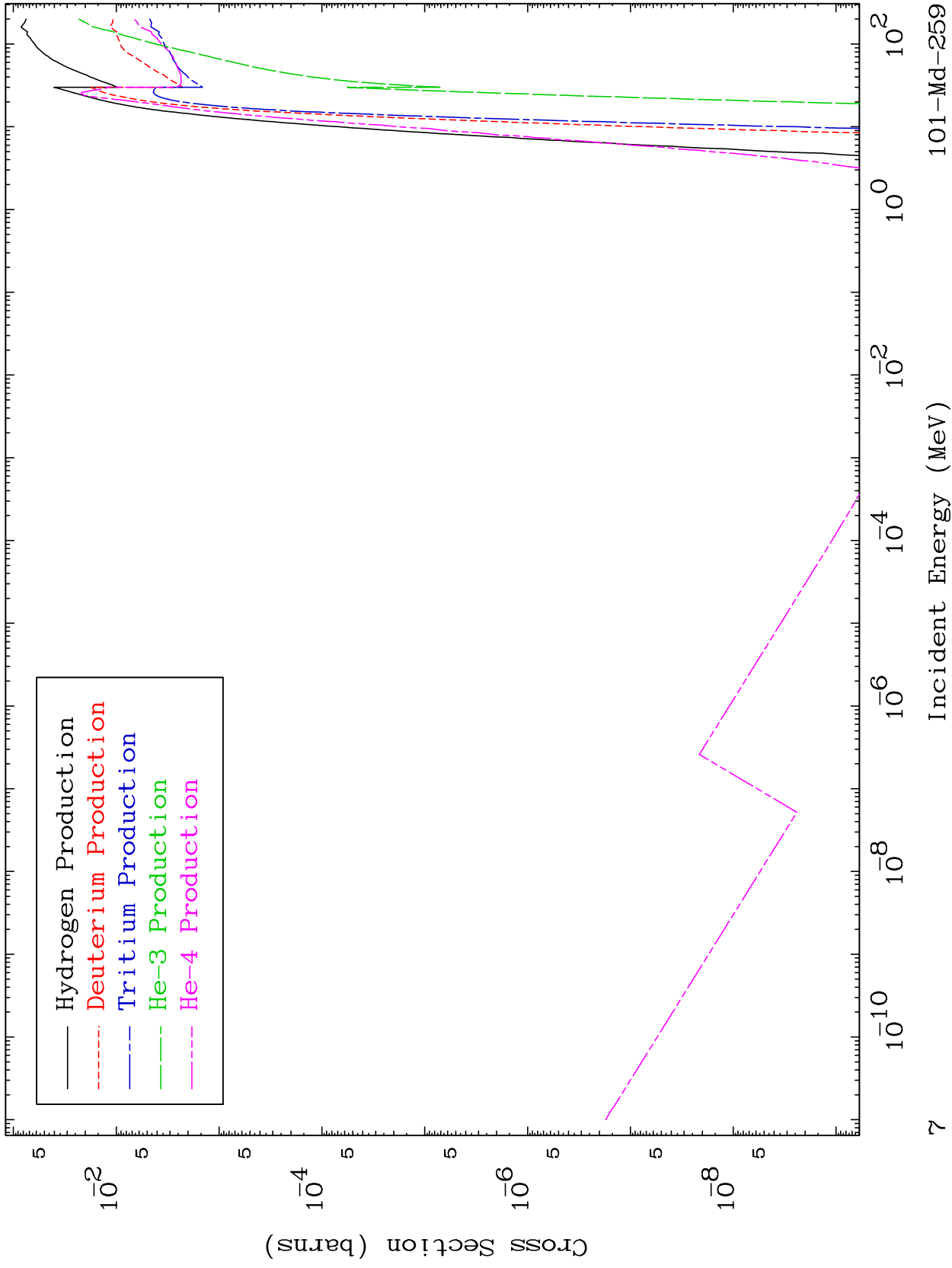
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Particle Production  
293 Kelvin Cross Sections

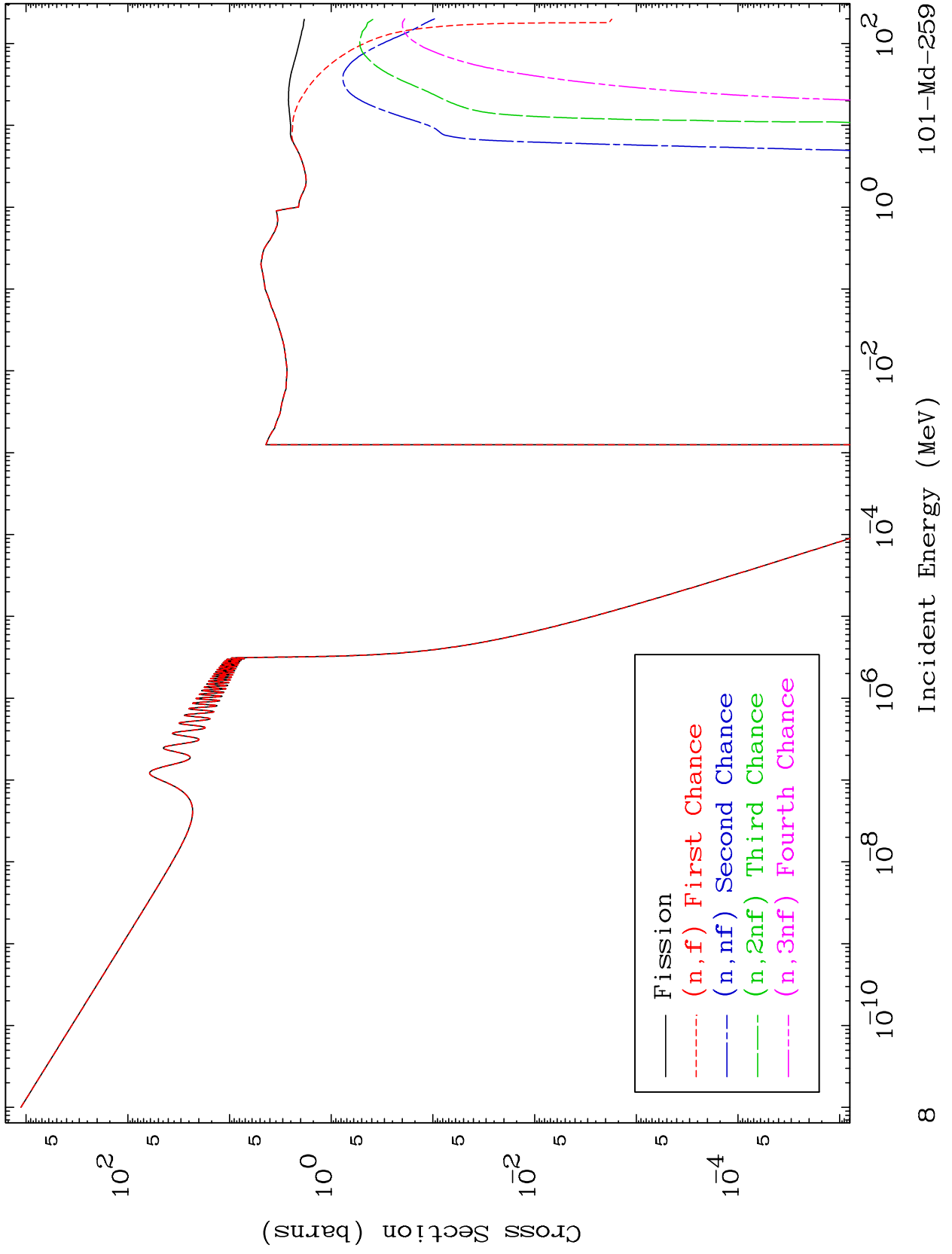
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Fission  
293 Kelvin Cross Sections

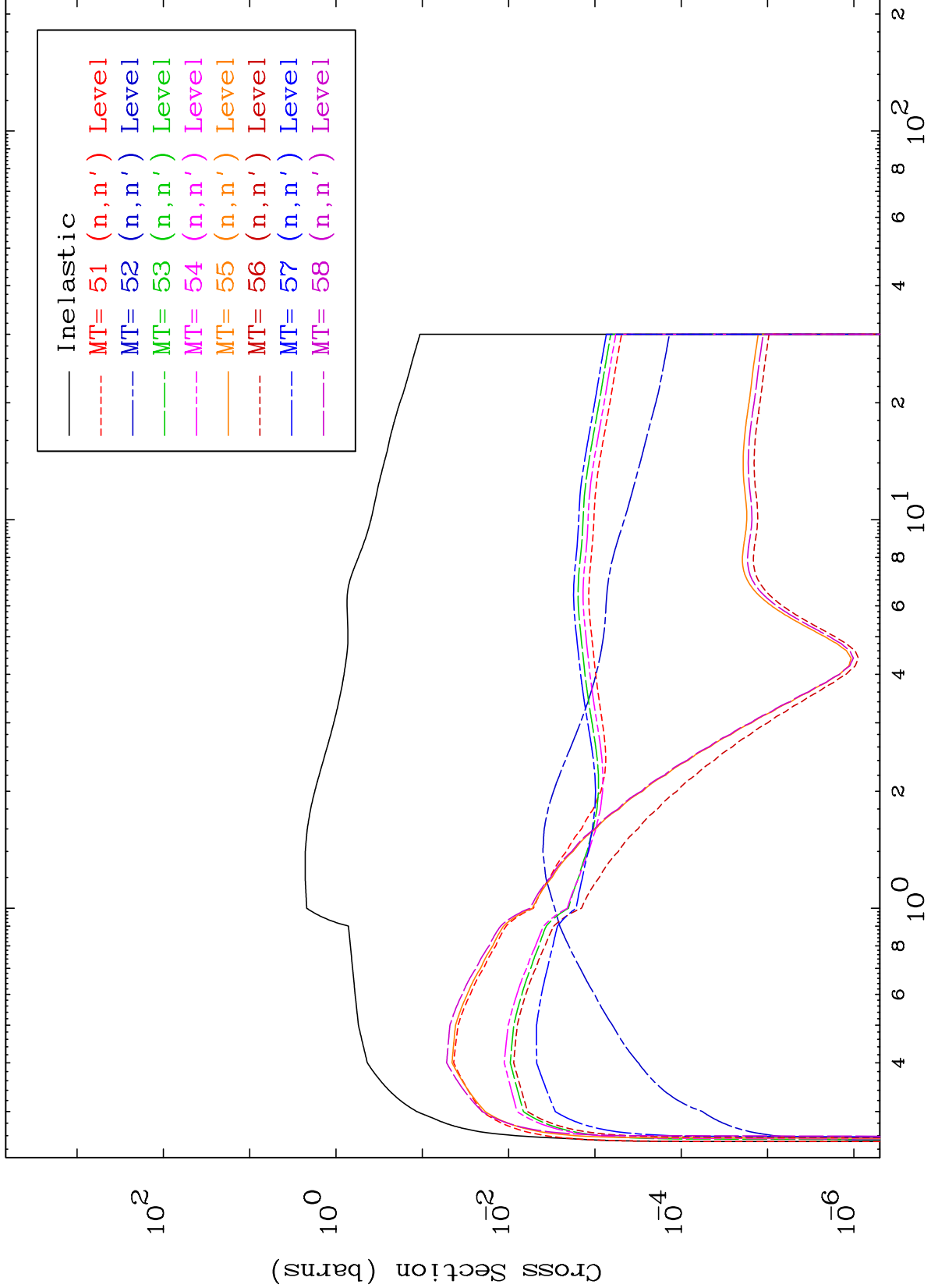
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(n,n') Levels  
293 Kelvin Cross Sections

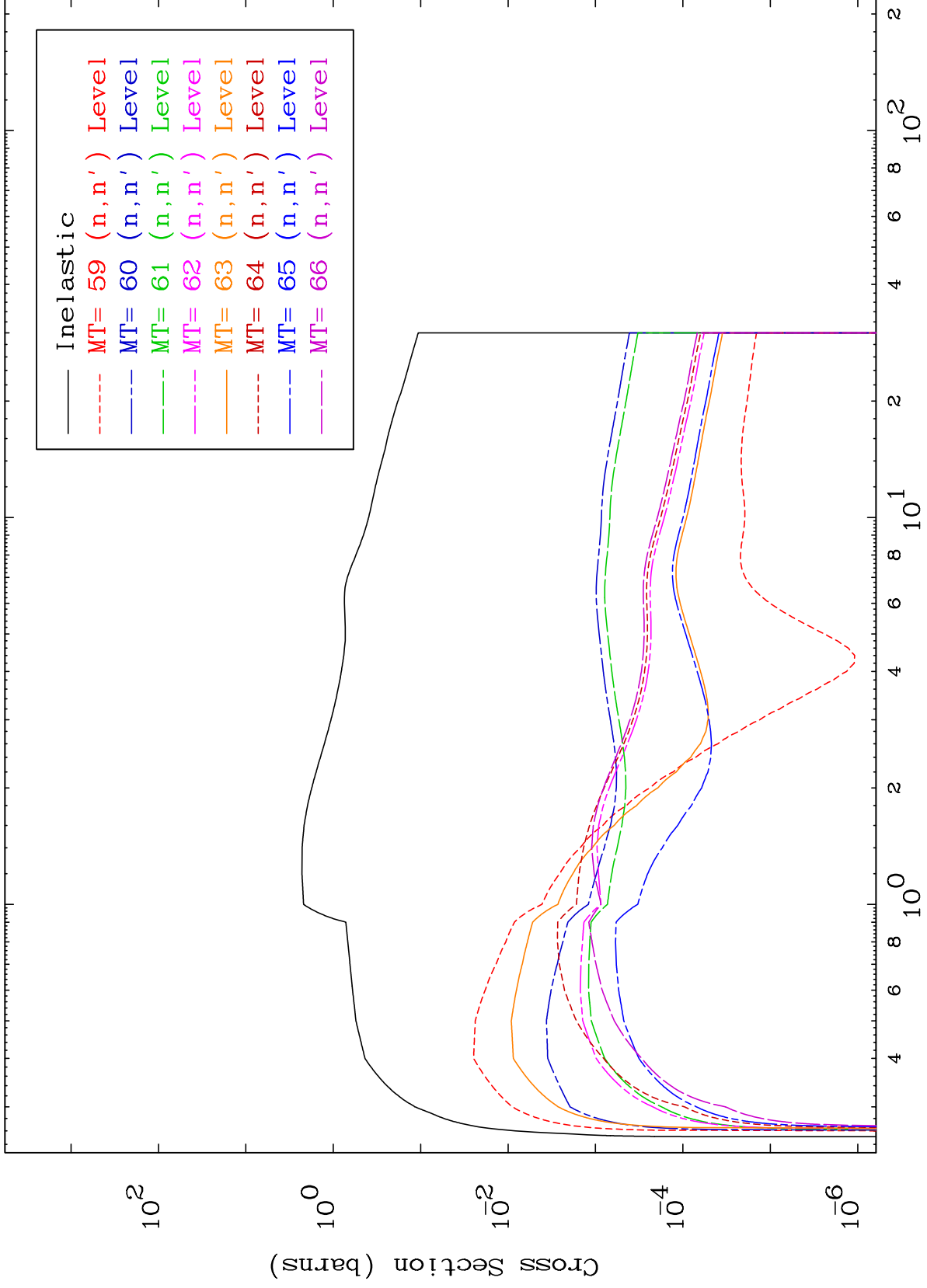
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MAT 9960

(n,n') Levels  
293 Kelvin Cross Sections

101-Md-259



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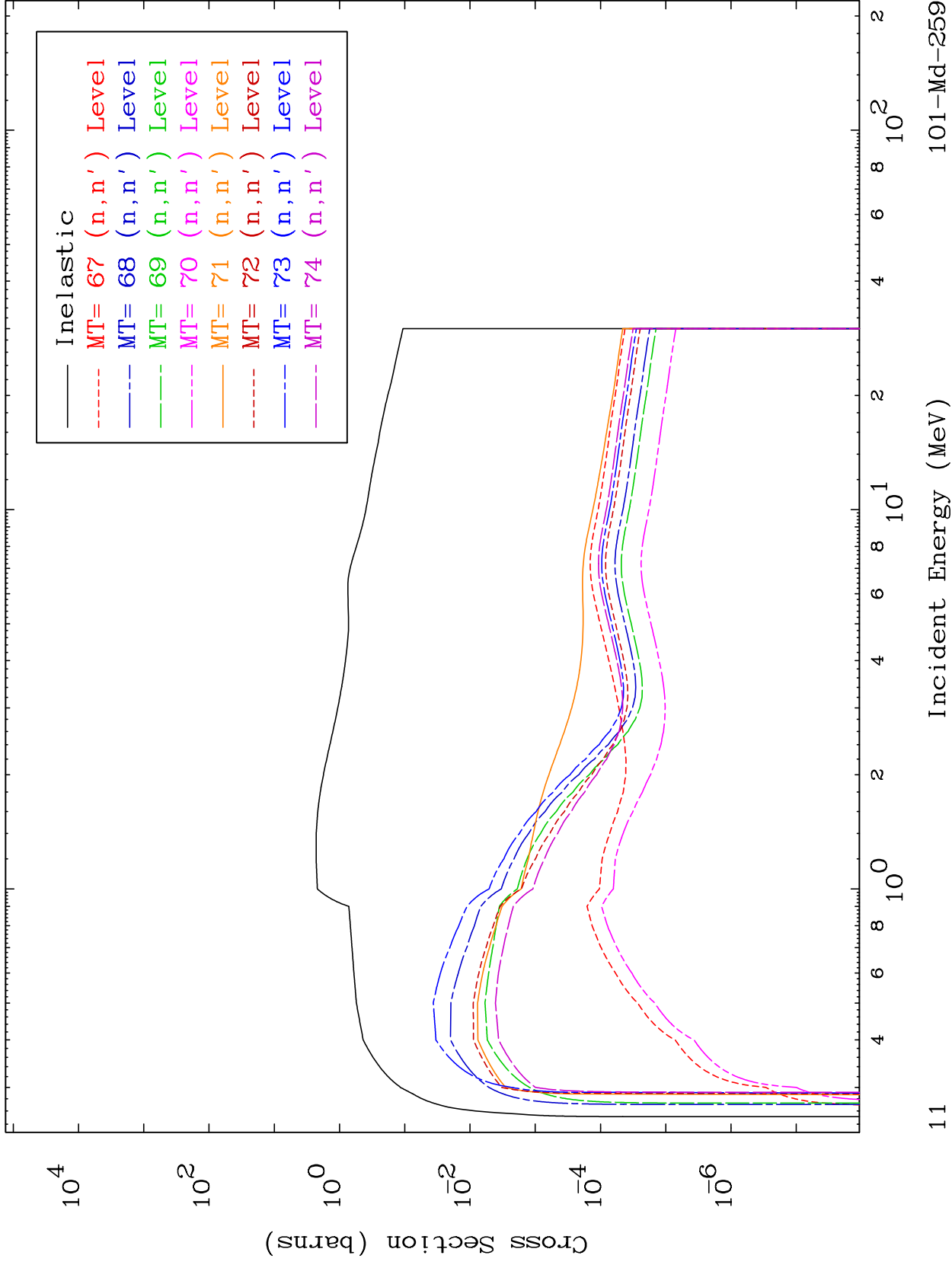
Incident Energy (MeV)

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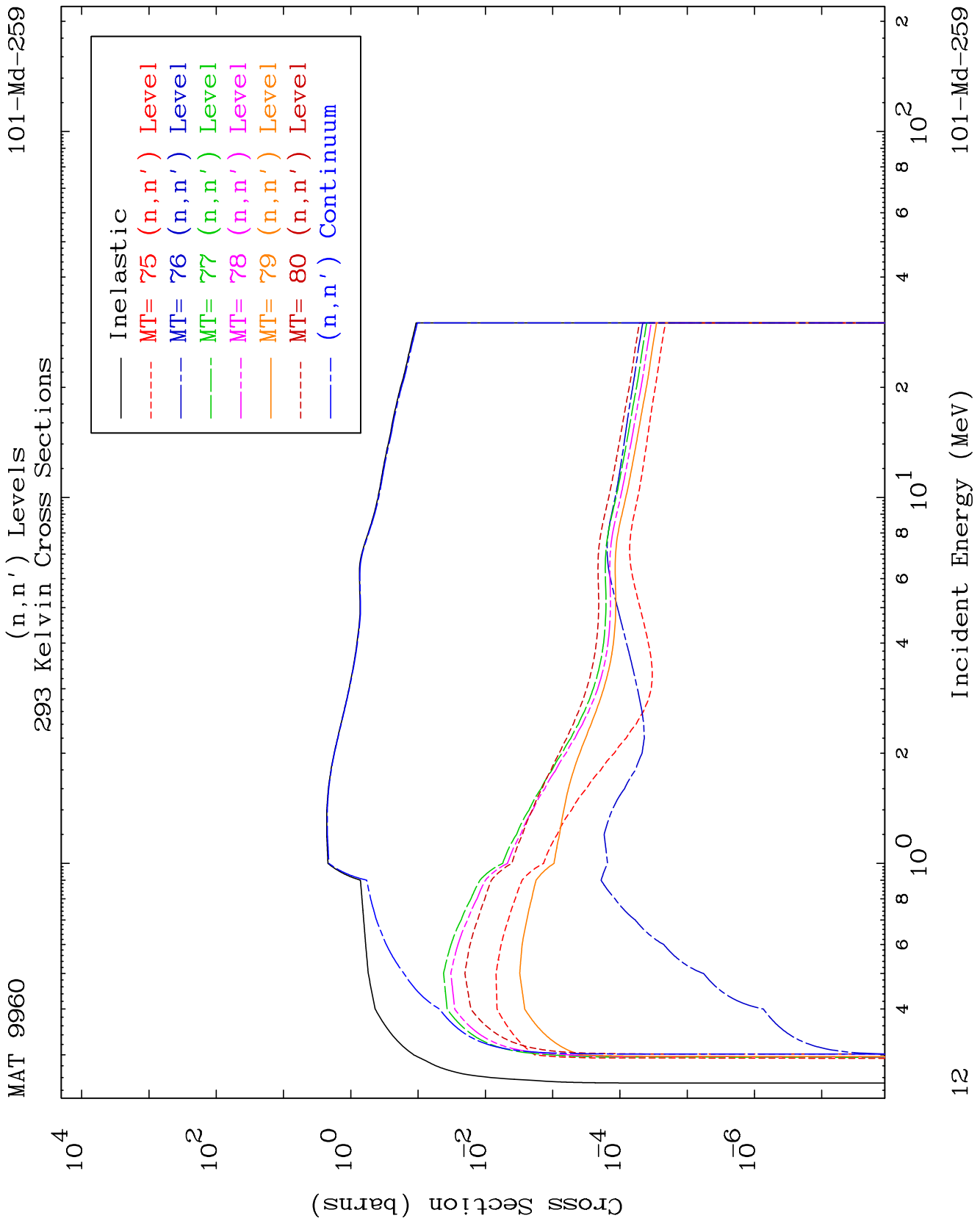
(n,n') Levels  
293 Kelvin Cross Sections

101-Md-259



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101-Md-259

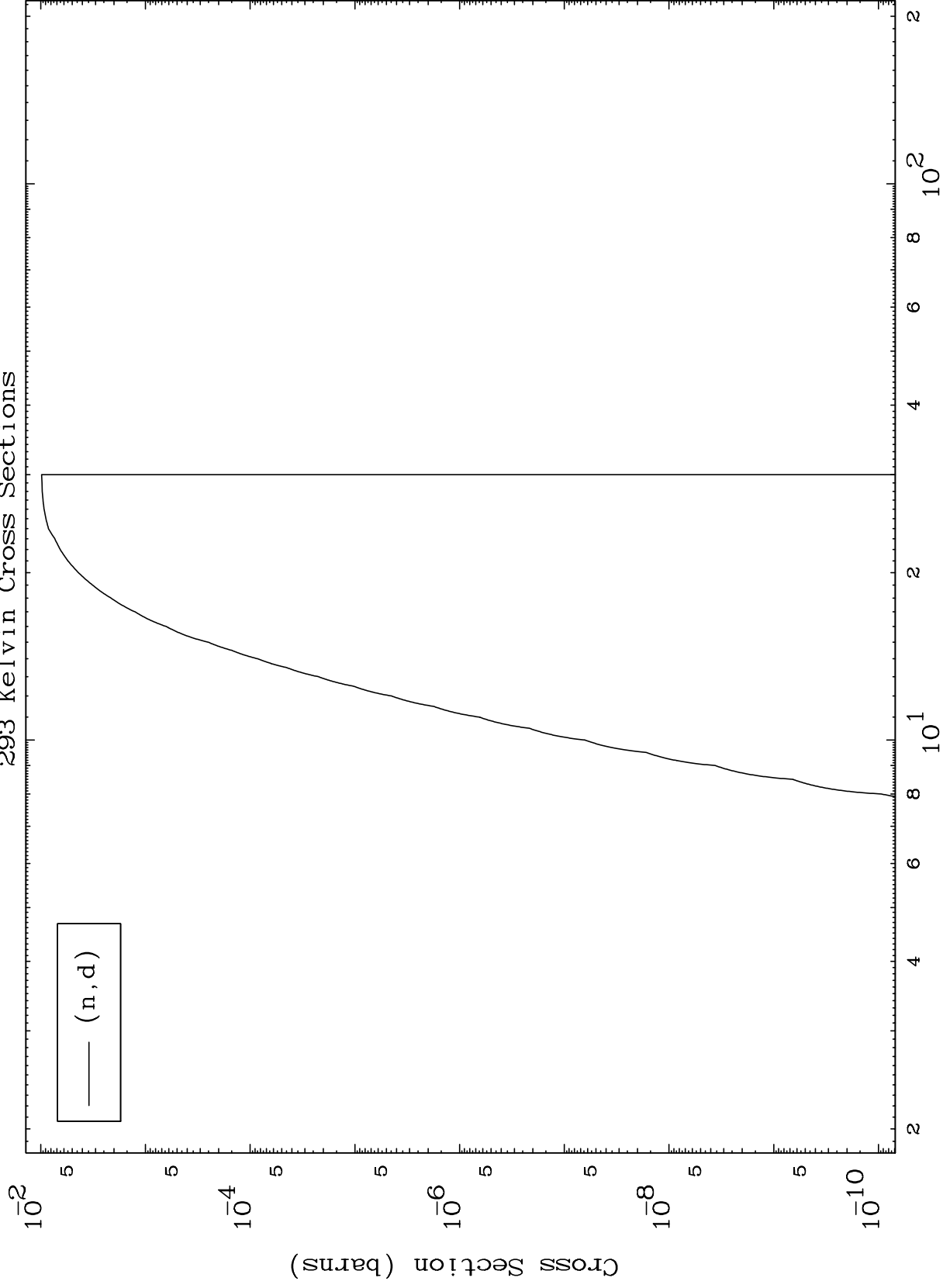




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(n,d) Levels  
293 Kelvin Cross Sections

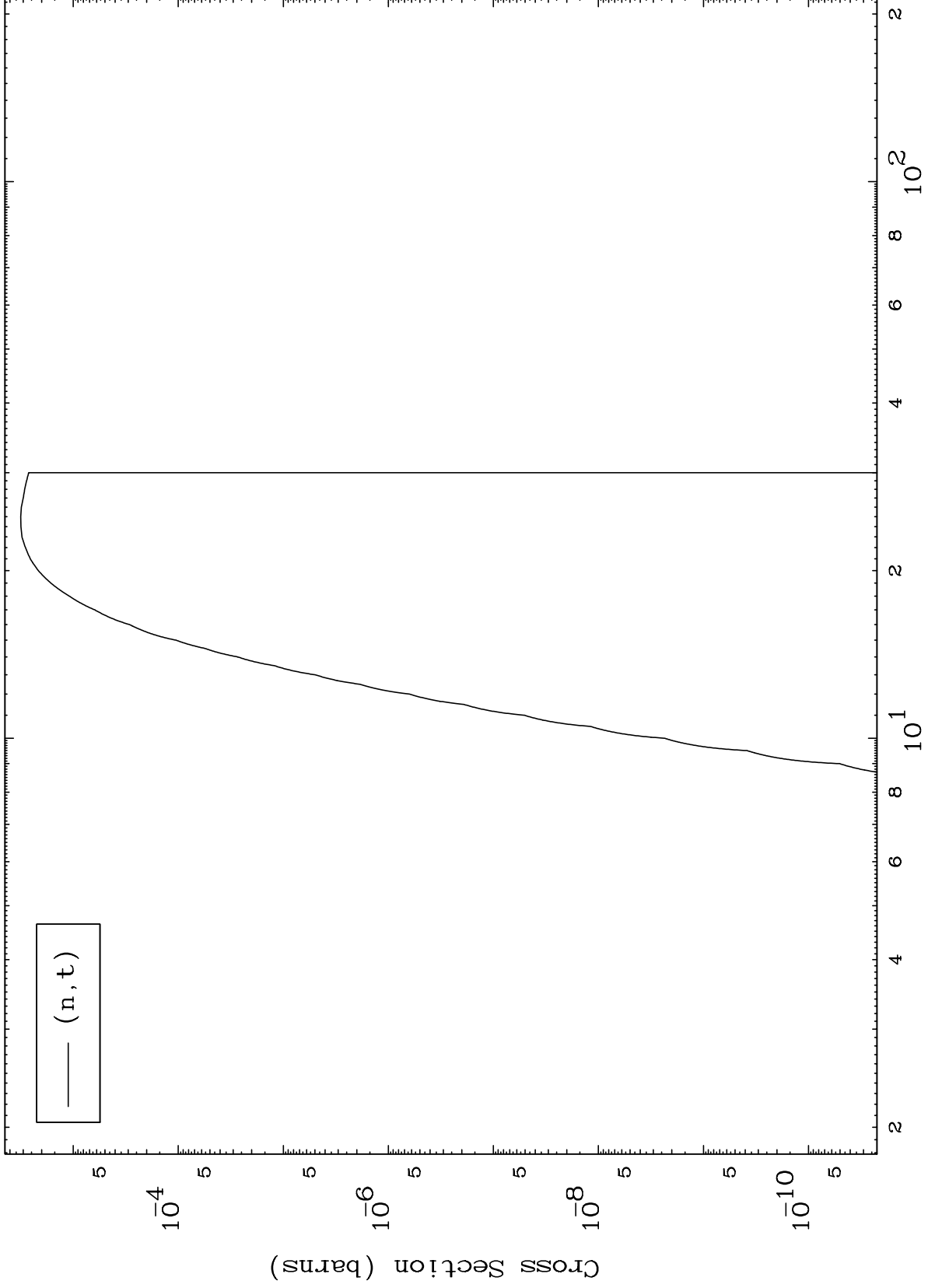
101-Md-259



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(n,t) Levels  
293 Kelvin Cross Sections

101-Md-259



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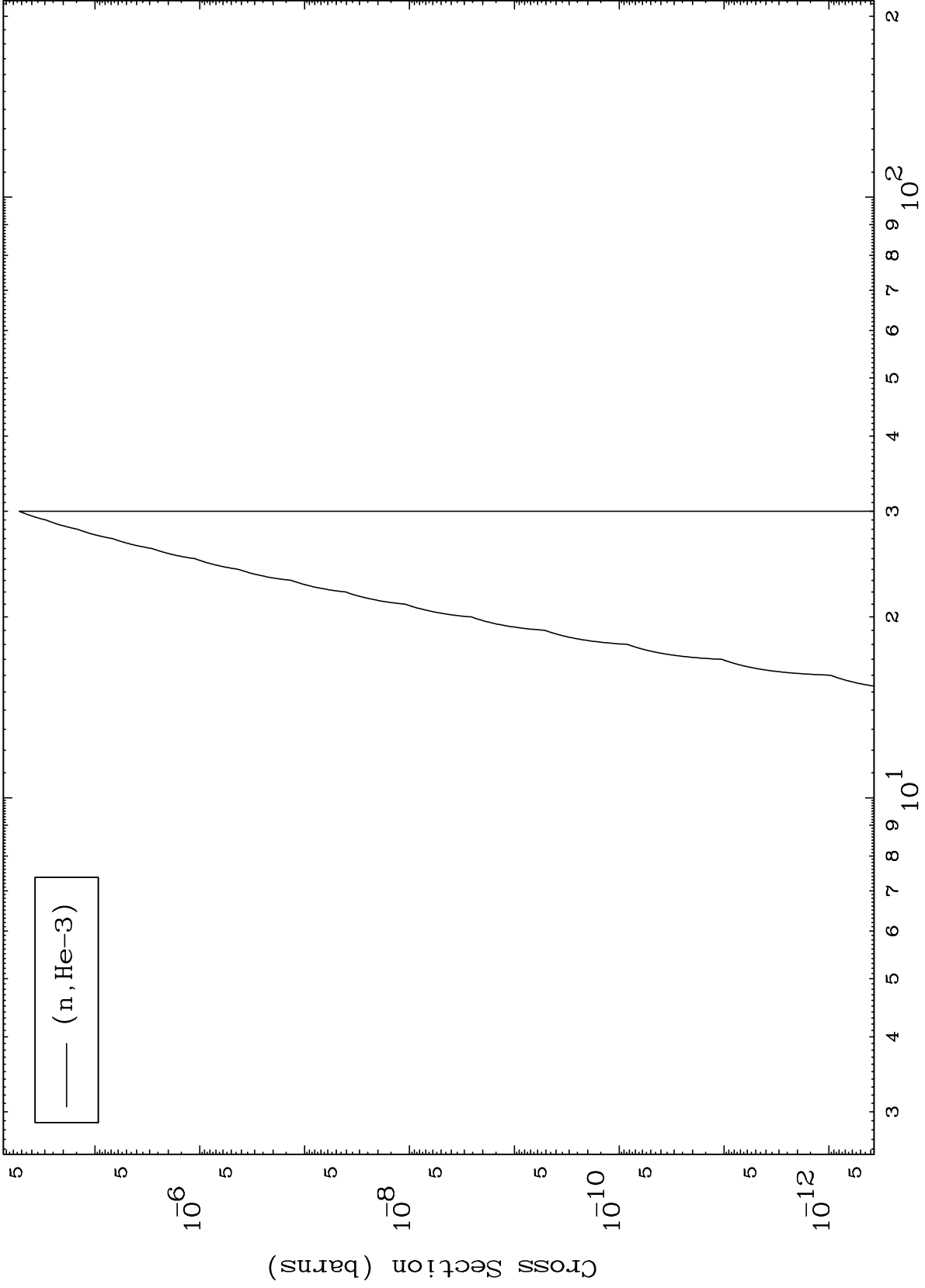
Incident Energy (MeV)

101-Md-259

MAT 9960

(n,He3) Levels  
293 Kelvin Cross Sections

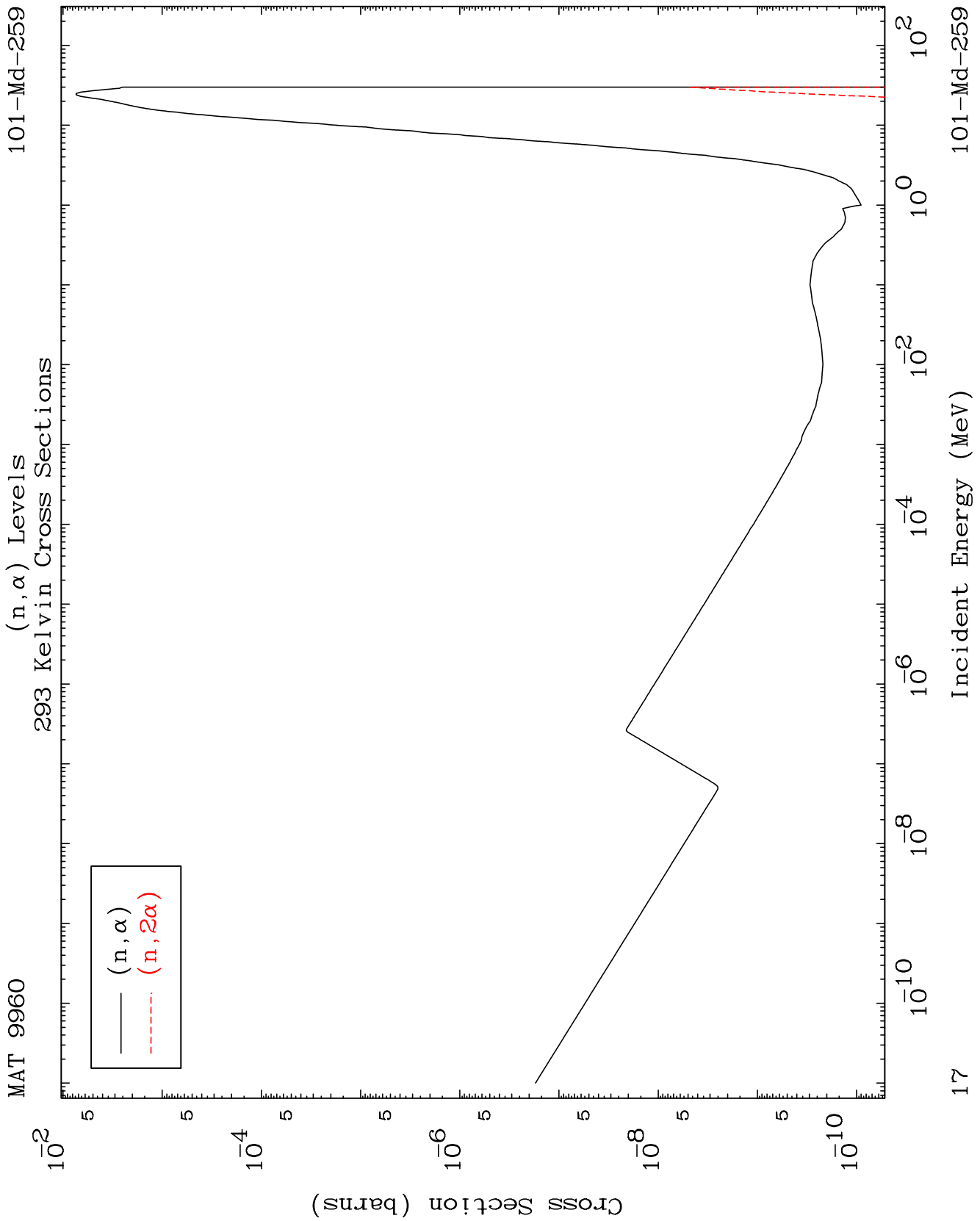
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Incident Energy (MeV)

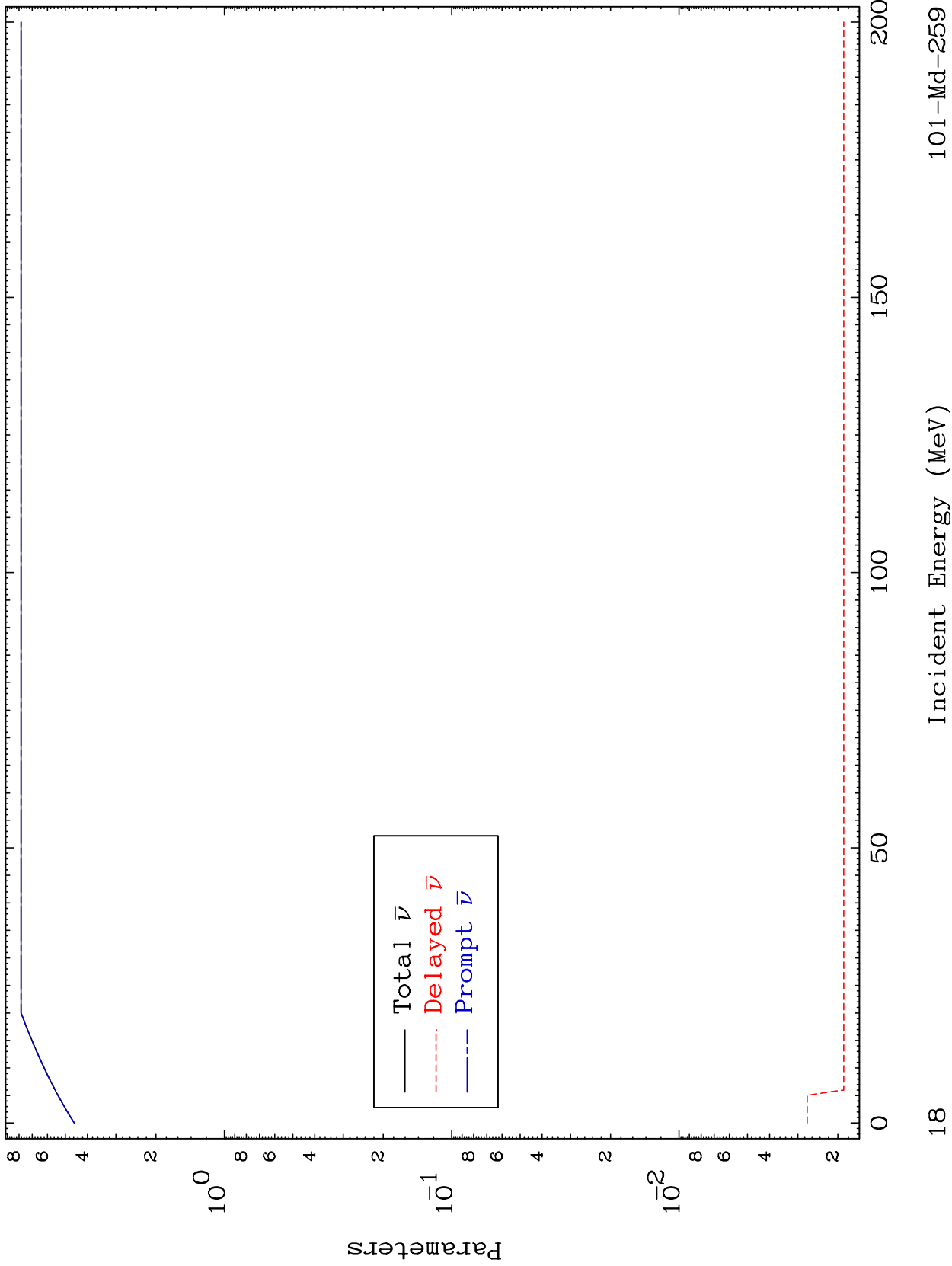
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Energy Release  
Parameters

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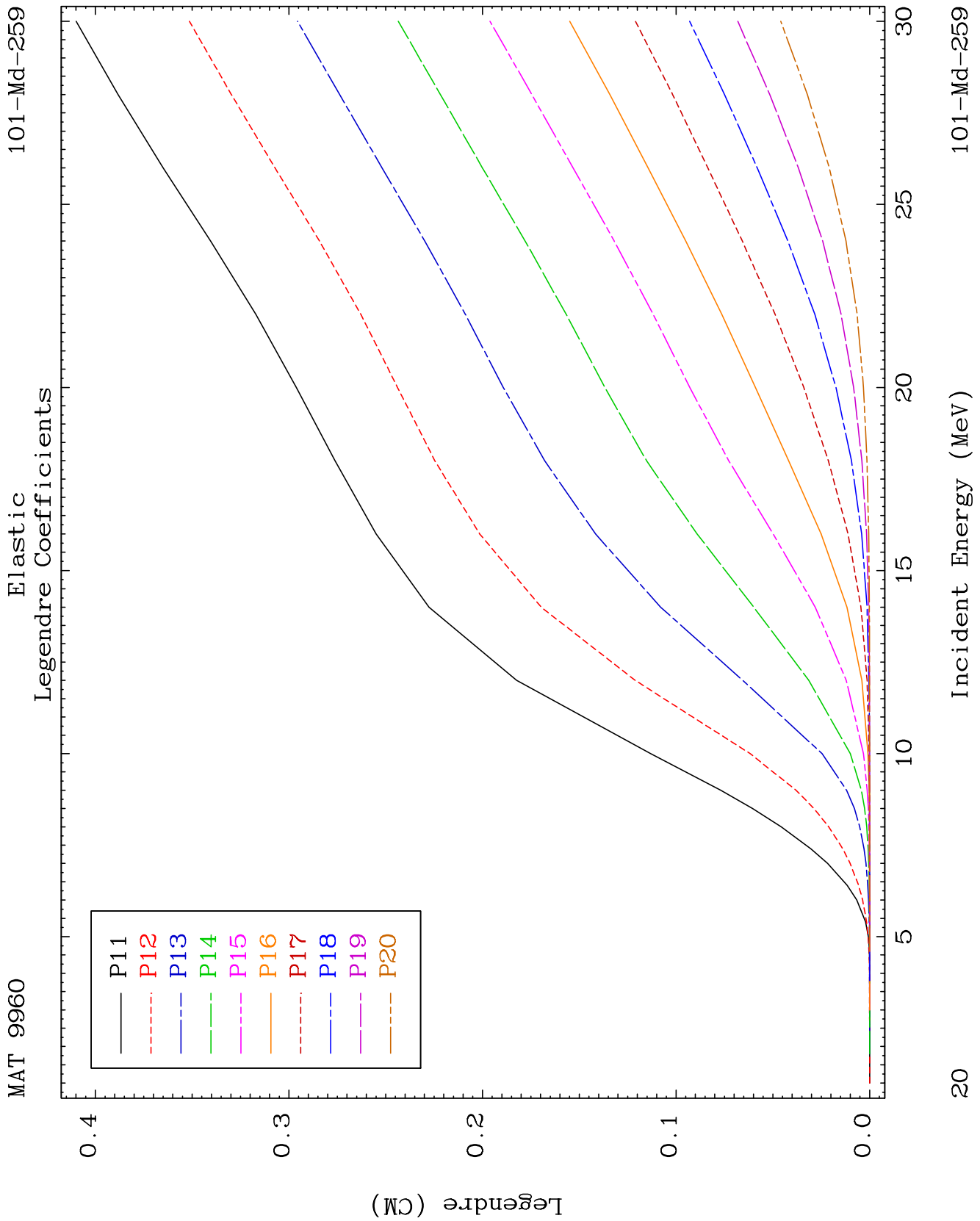


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Incident Energy (MeV)

101-Md-259

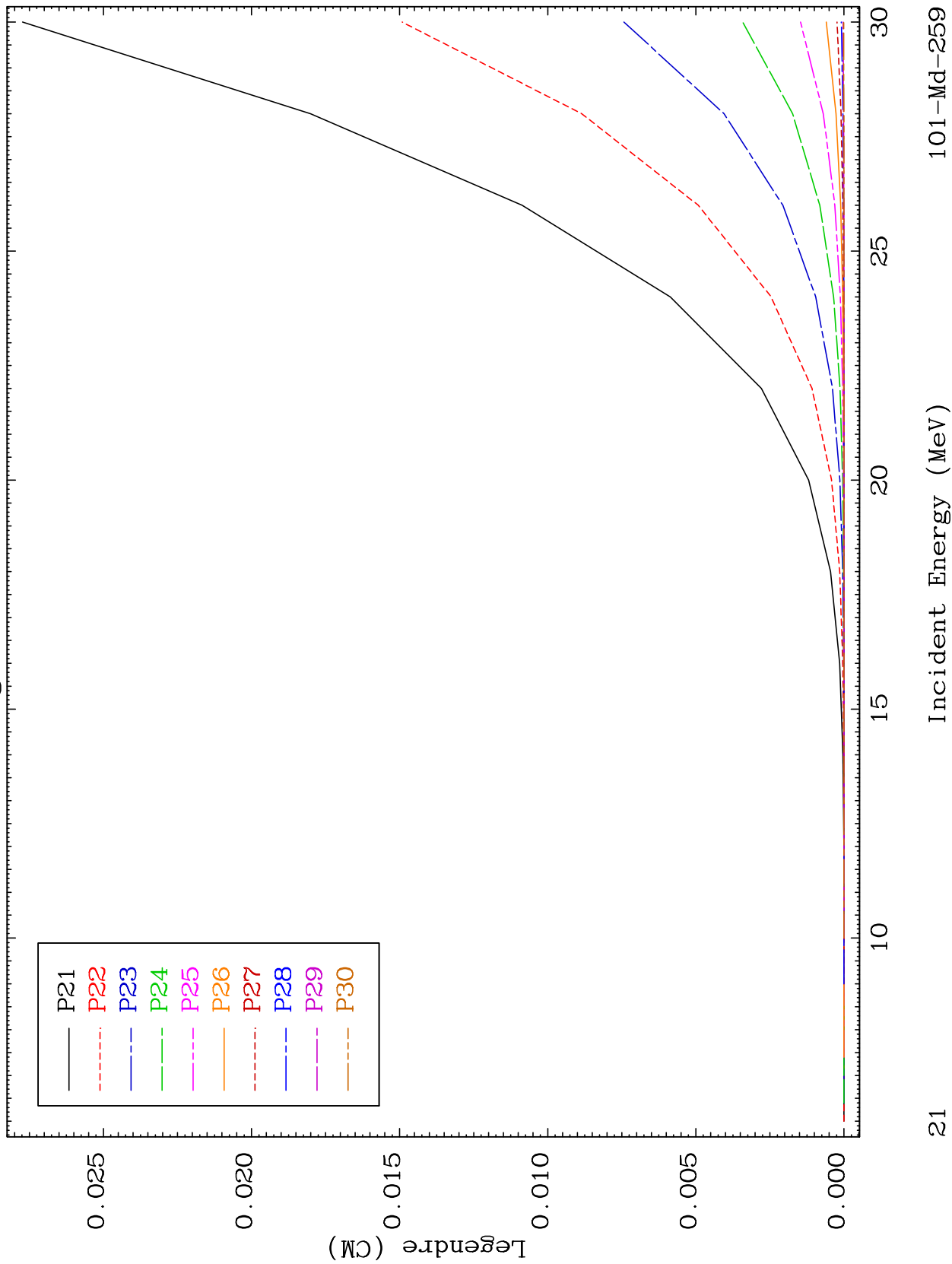




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### Elastic Legendre Coefficients

101-Md-259

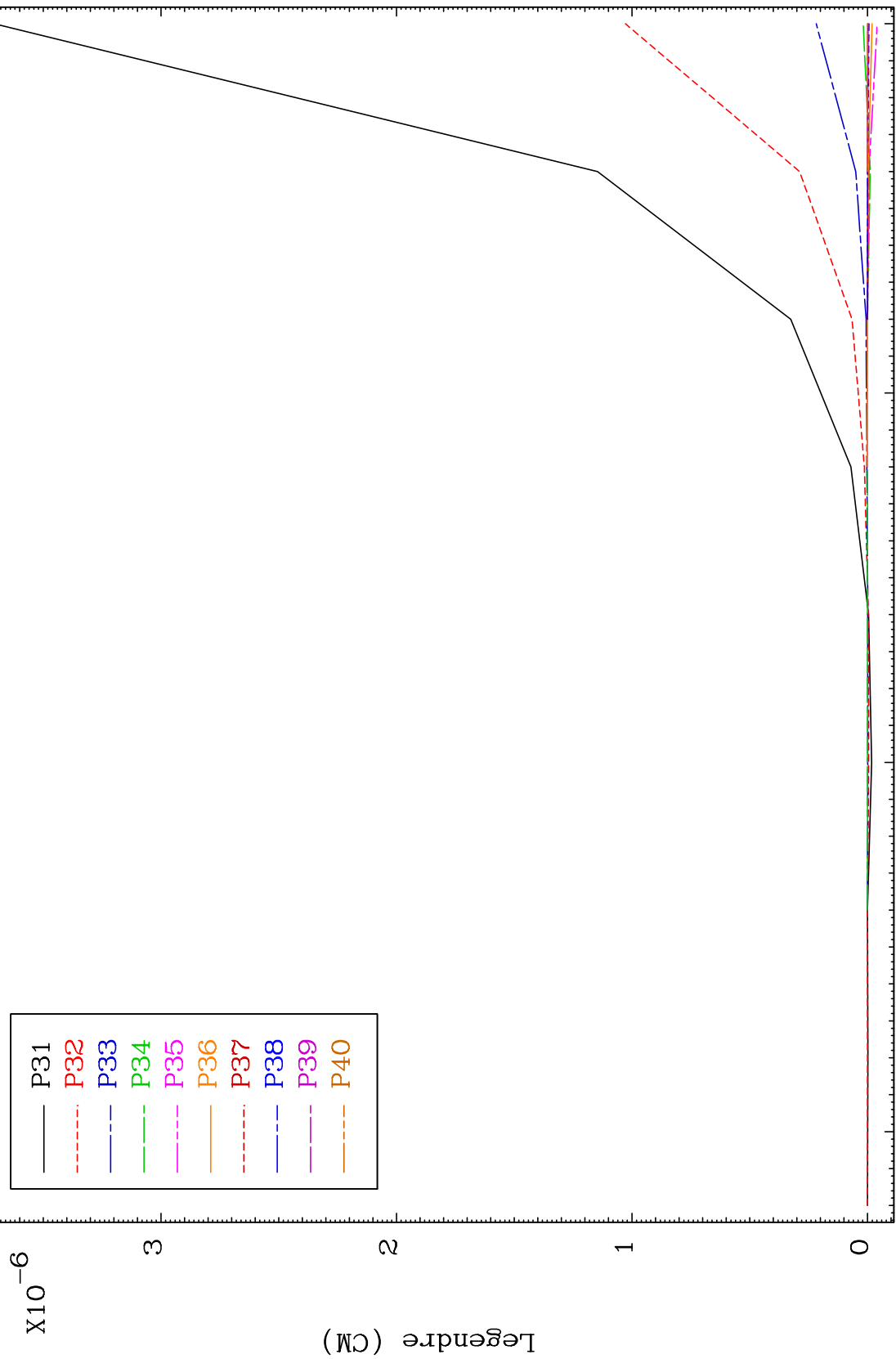
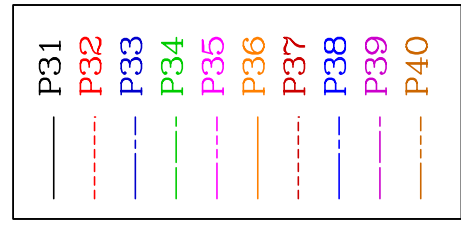


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Elastic Legendre Coefficients

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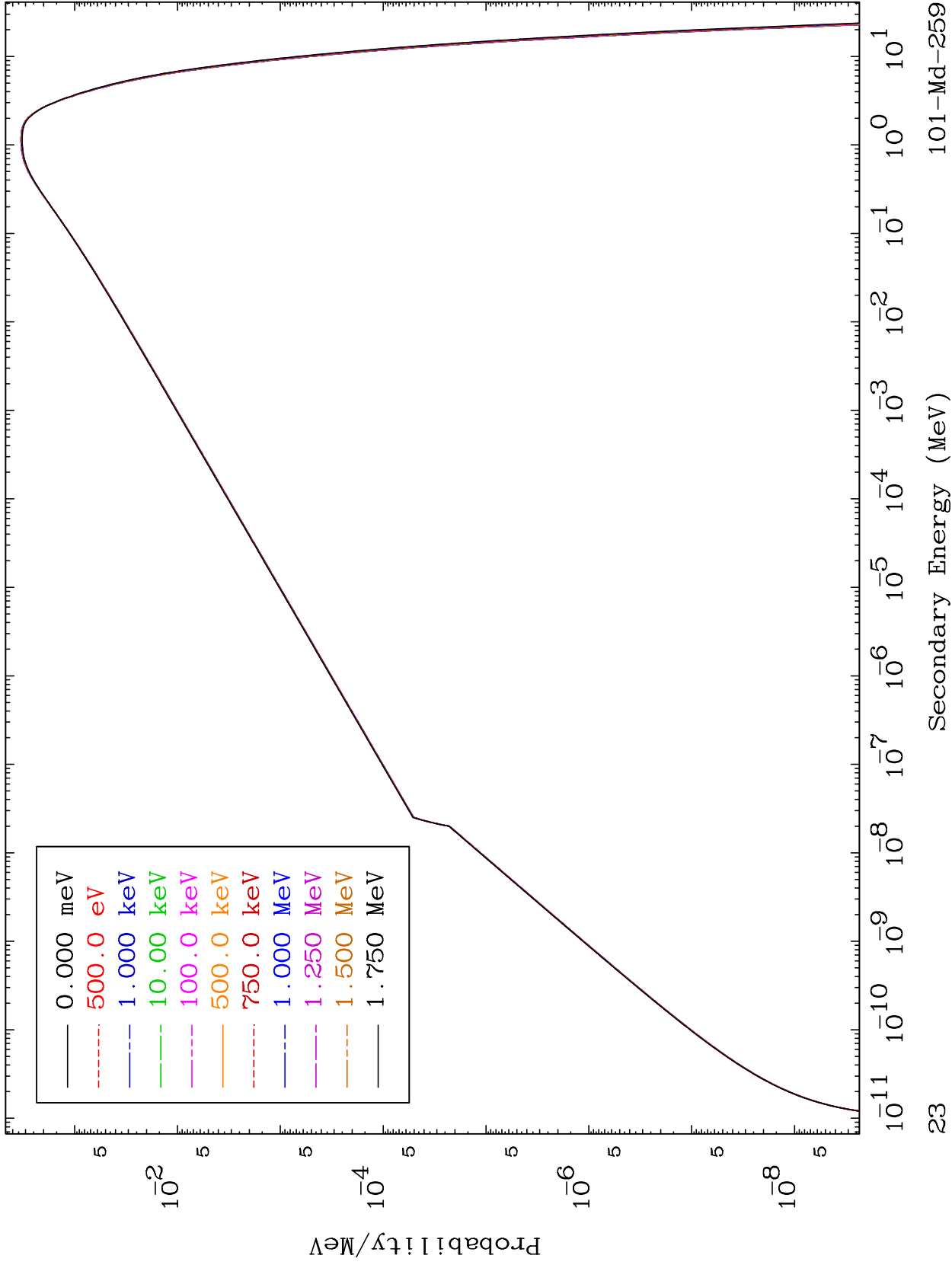
Incident Energy (MeV)

101-Md-259

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Fission Energy Distributions

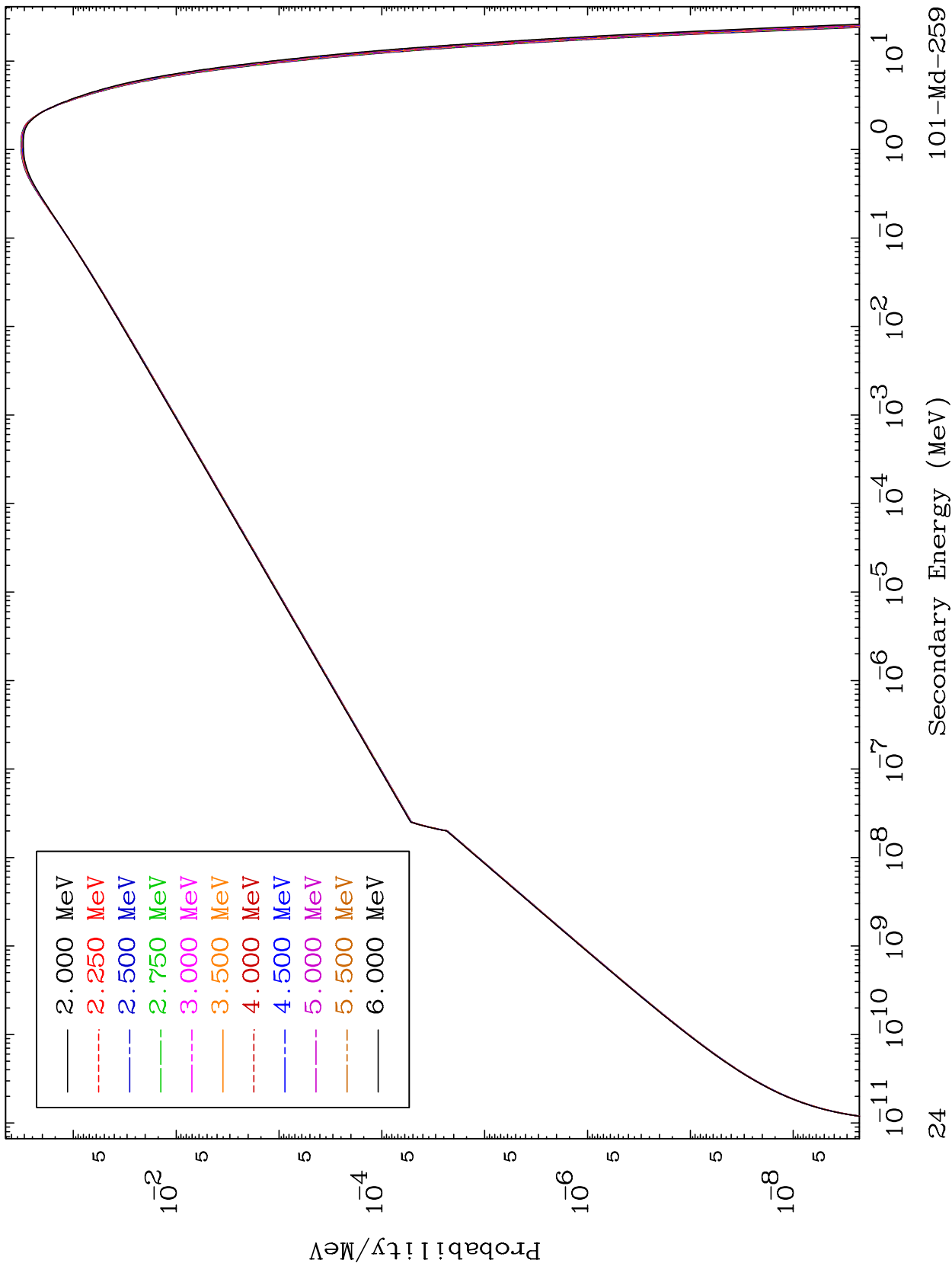
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### Fission Energy Distributions

101-Md-259



101-Md-259

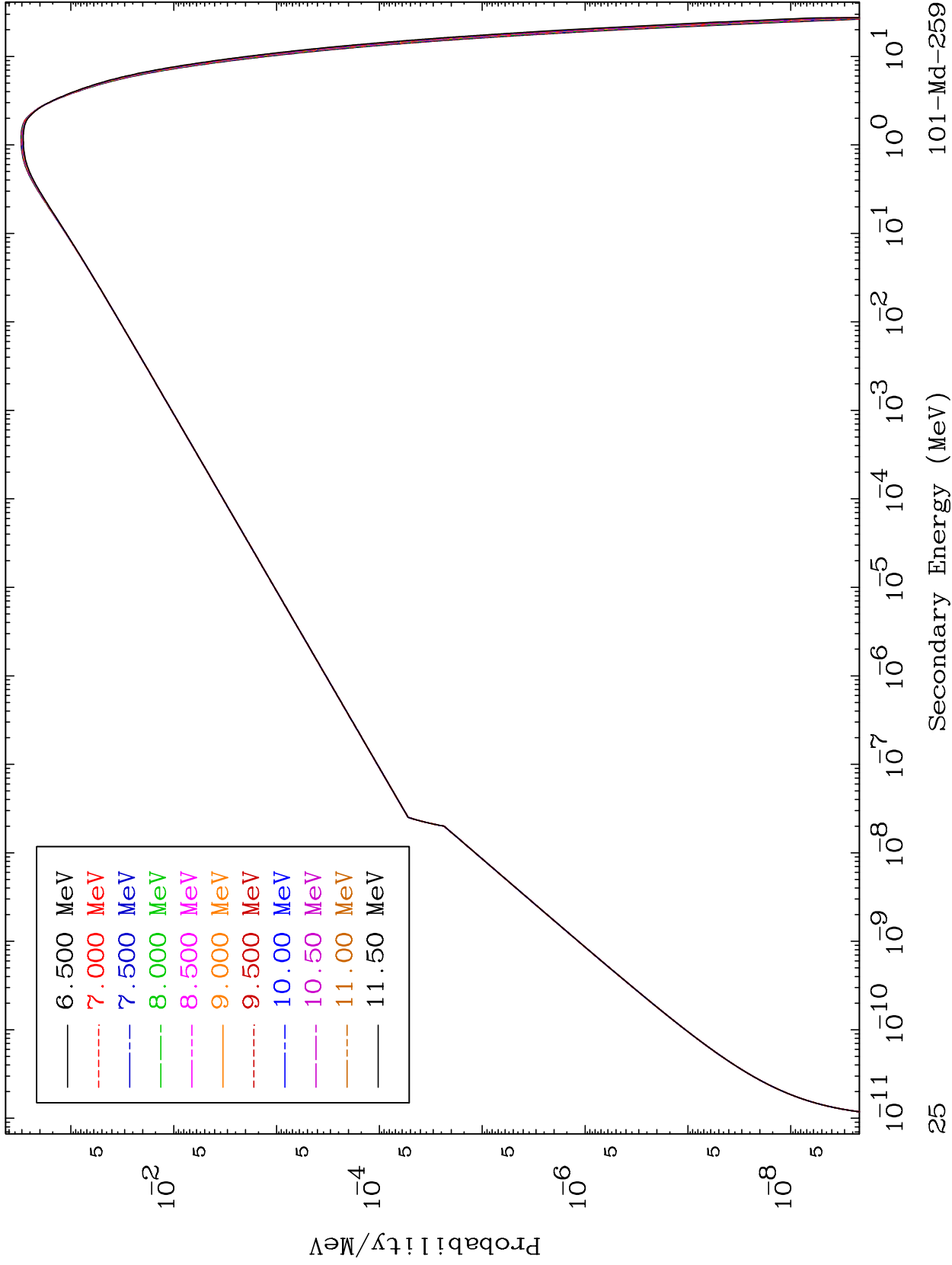
Secondary Energy (MeV)

24

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Fission Energy Distributions

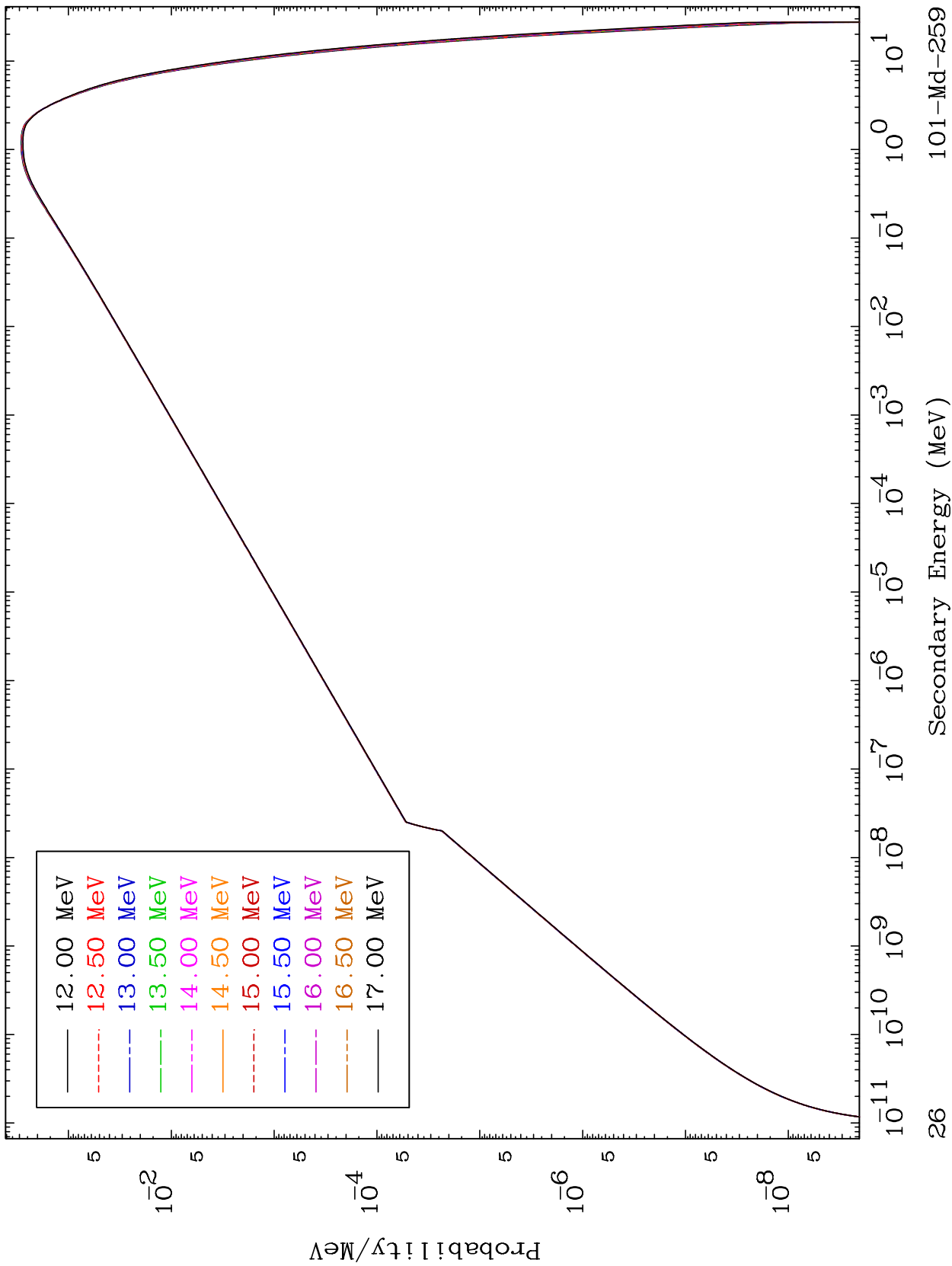
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### Fission Energy Distributions

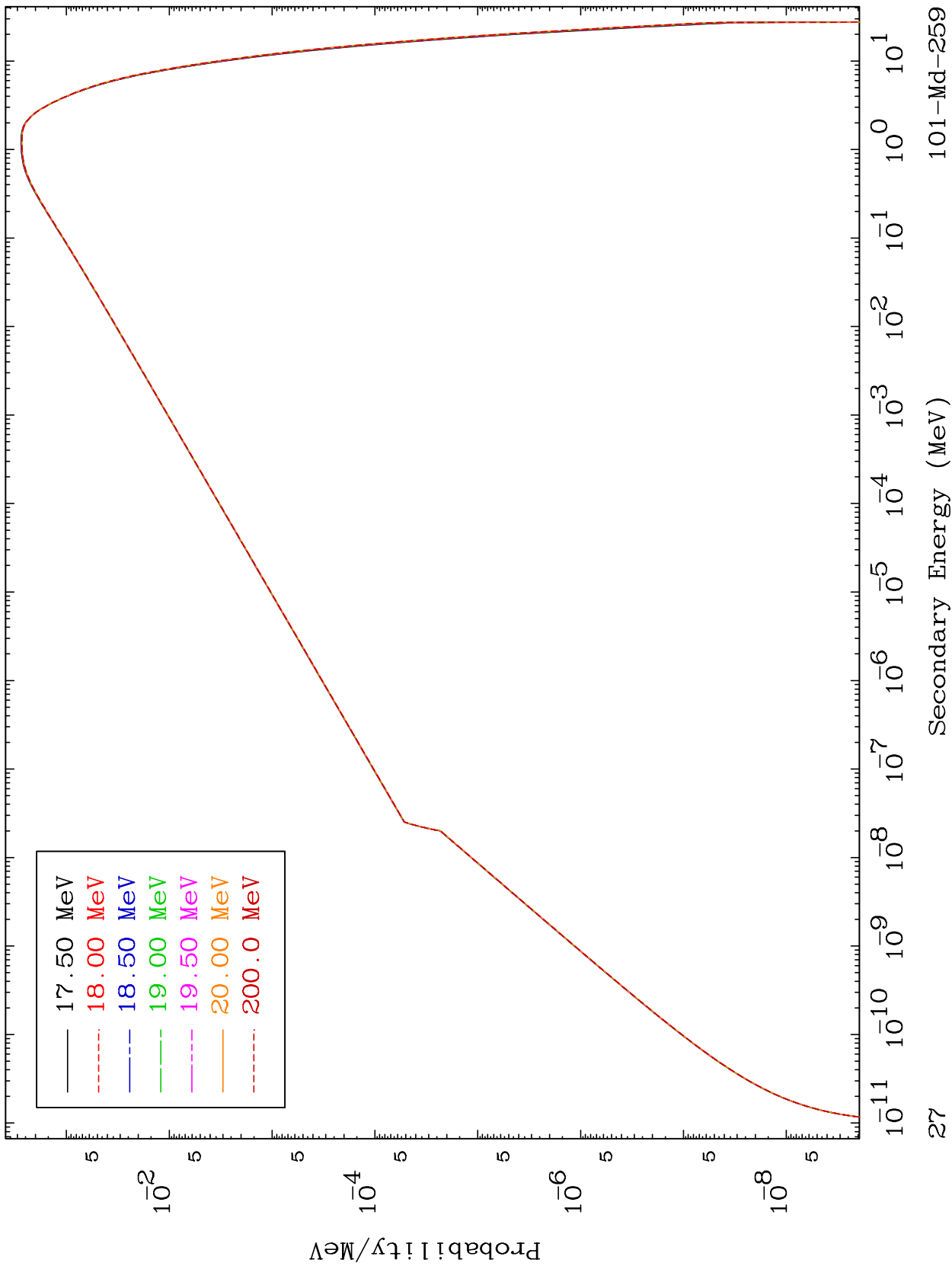
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### Fission Energy Distributions

101-Md-259

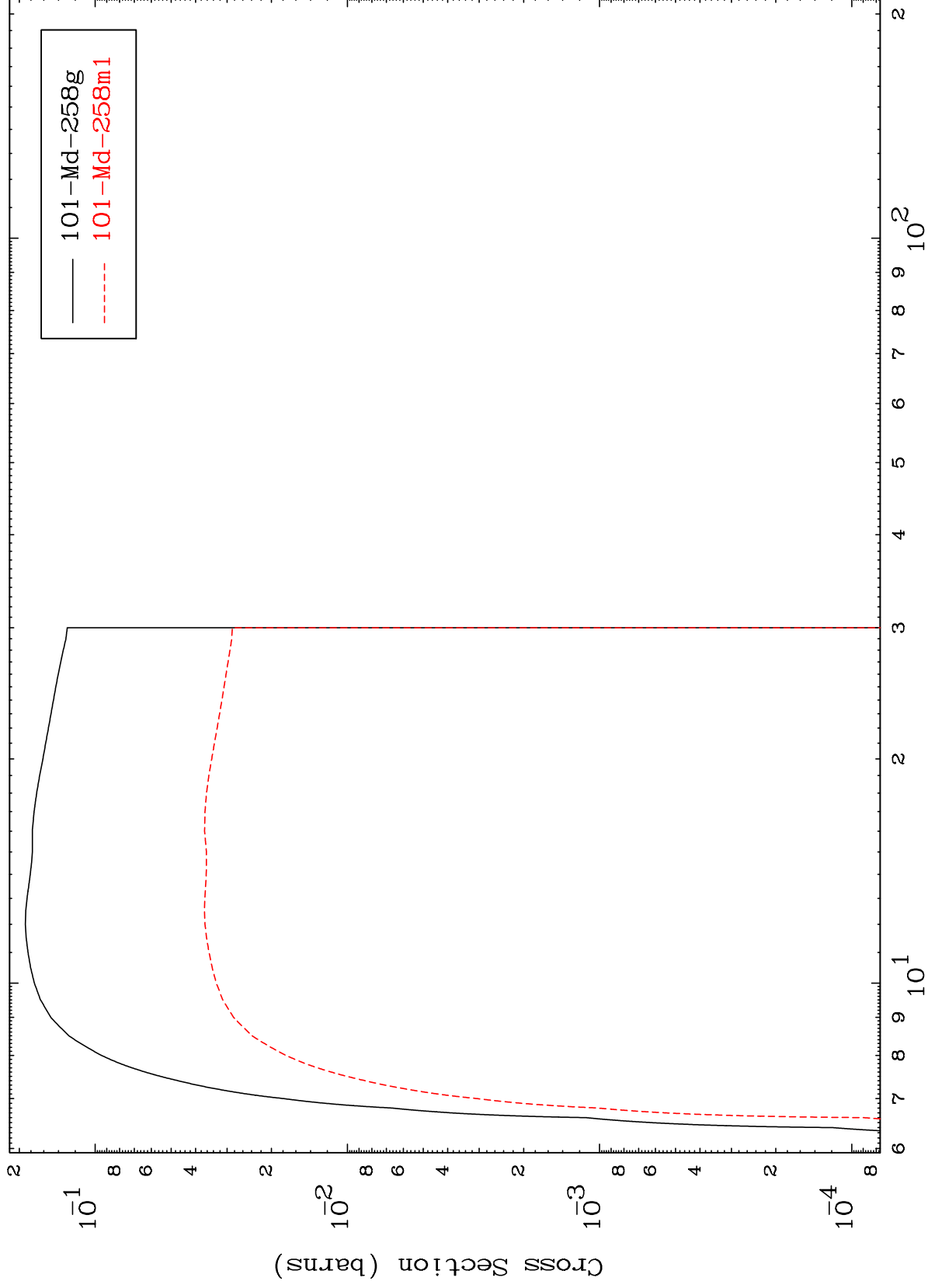


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(n,2n)

101-Md-259

Radionuclide Production Cross Section



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Incident Energy (MeV)

101-Md-259

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(n,2n)  $\alpha$

101-Md-259

Radionuclide Production Cross Section

