

Program EVALPLOT
(Version 2021-1)

by

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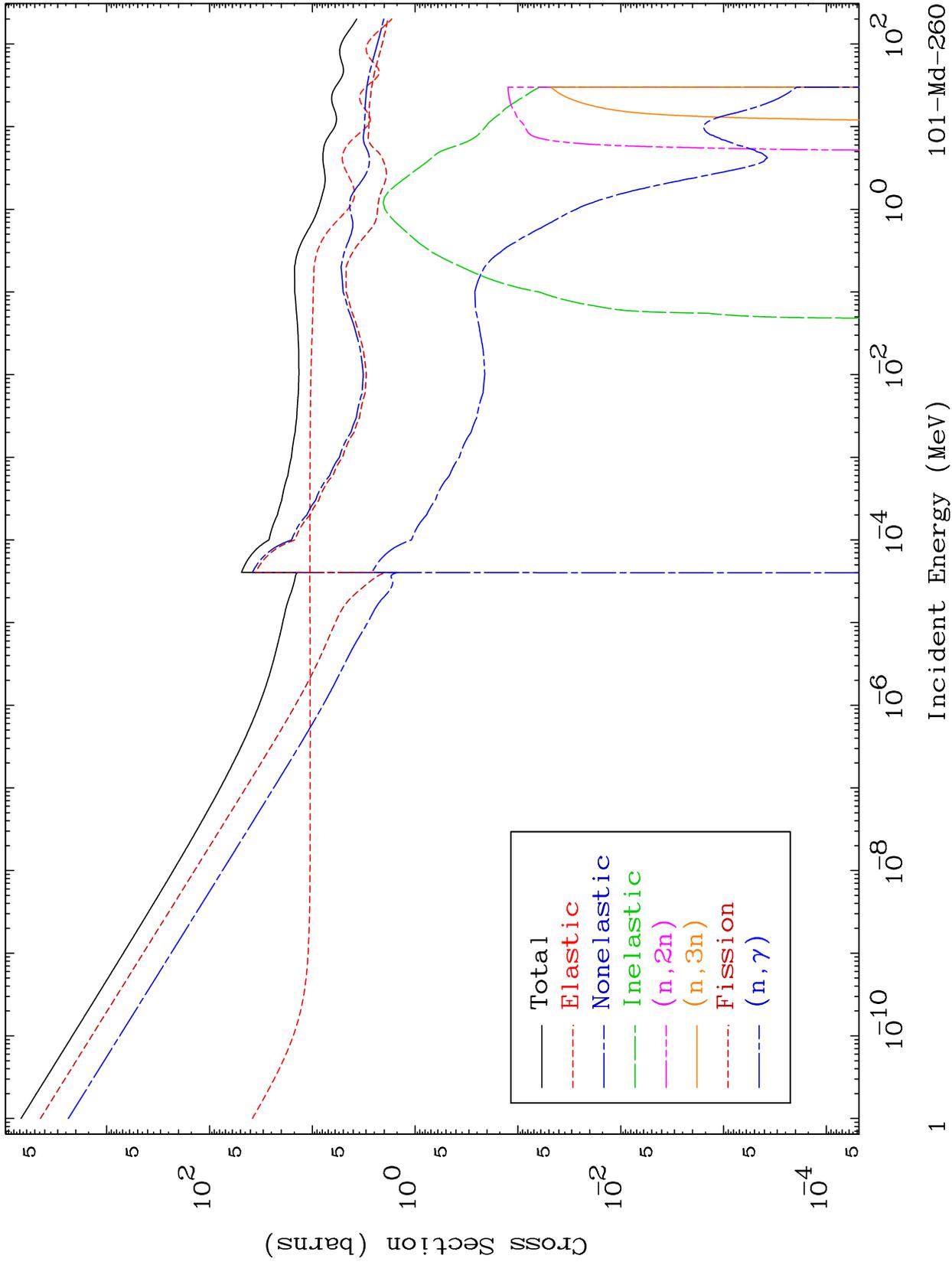
E.Mail:redcullen1@comcast.net
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9961

Neutron Major
293 Kelvin Cross Sections

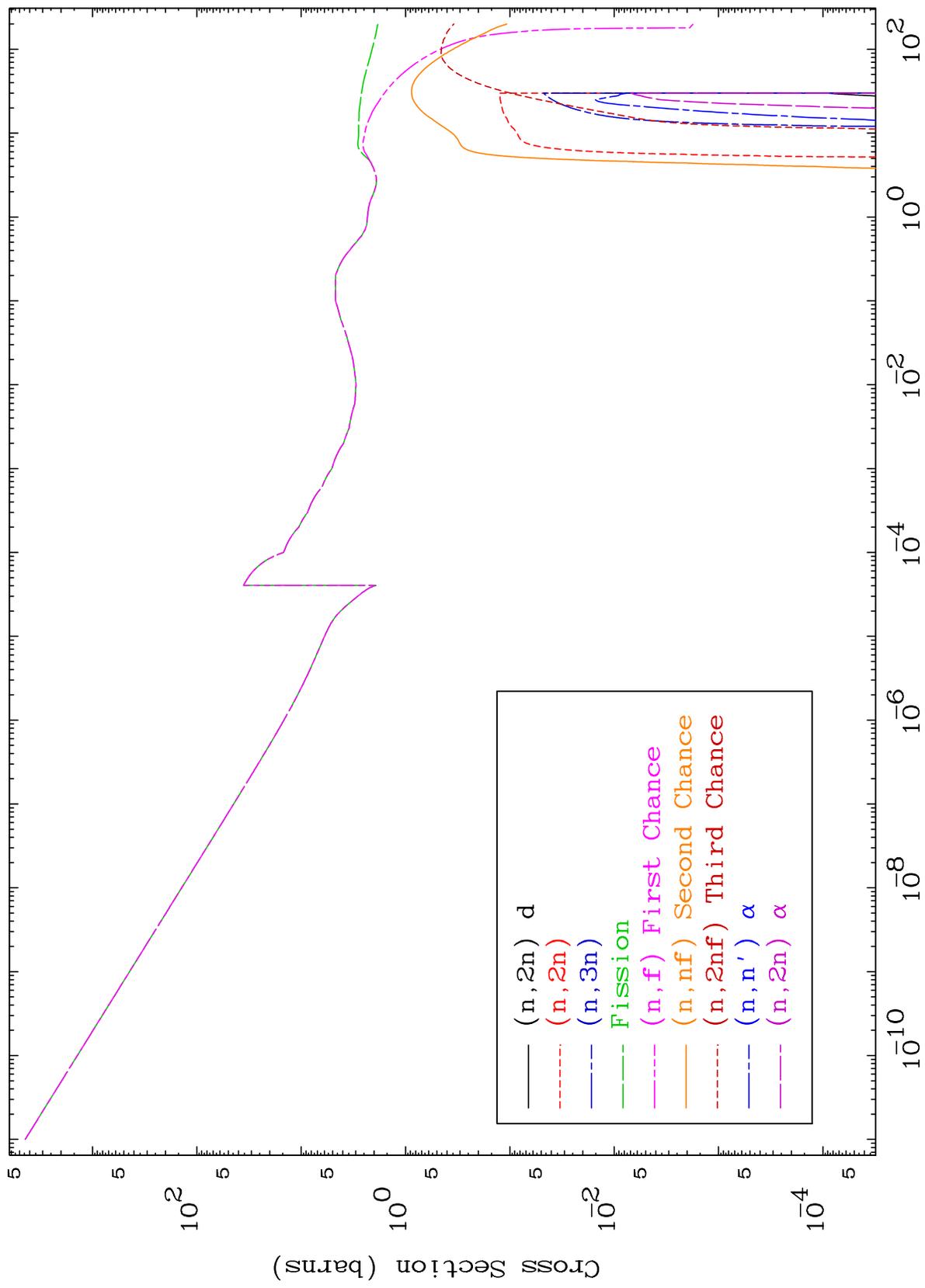
101-Md-260



MAT 9961

Neutron Absorption
293 Kelvin Cross Sections

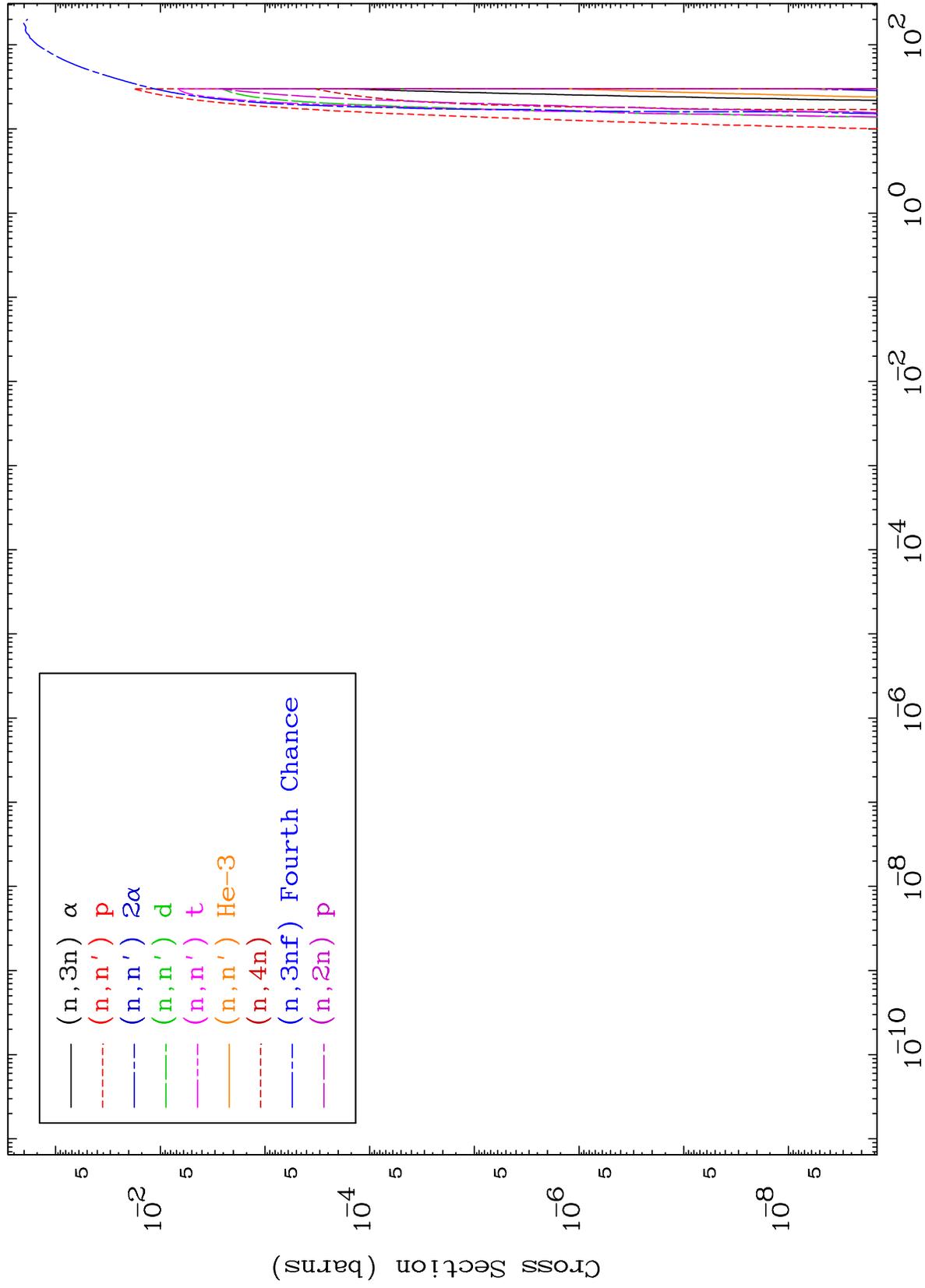
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Neutron Absorption 293 Kelvin Cross Sections

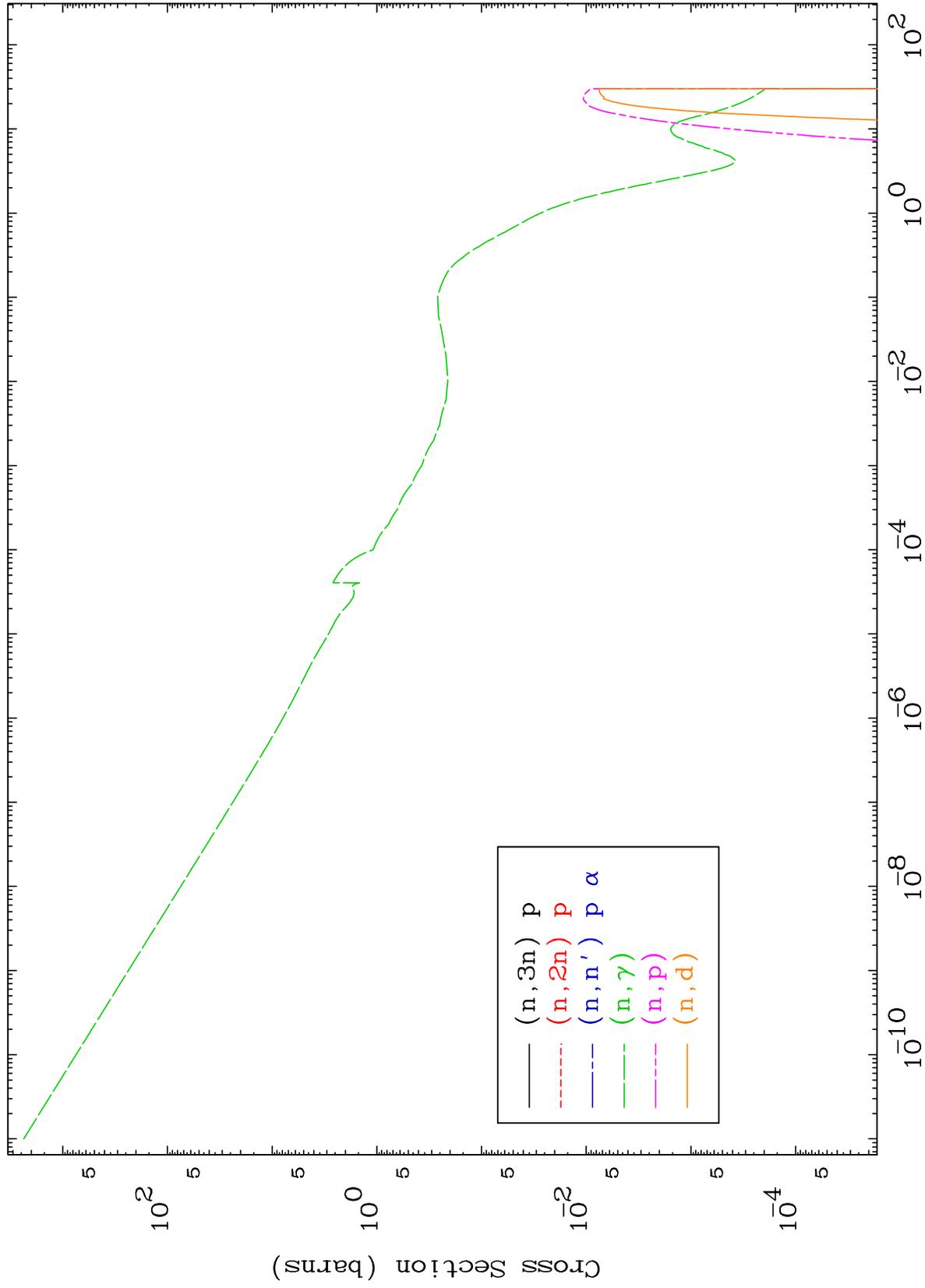
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Neutron Absorption
293 Kelvin Cross Sections

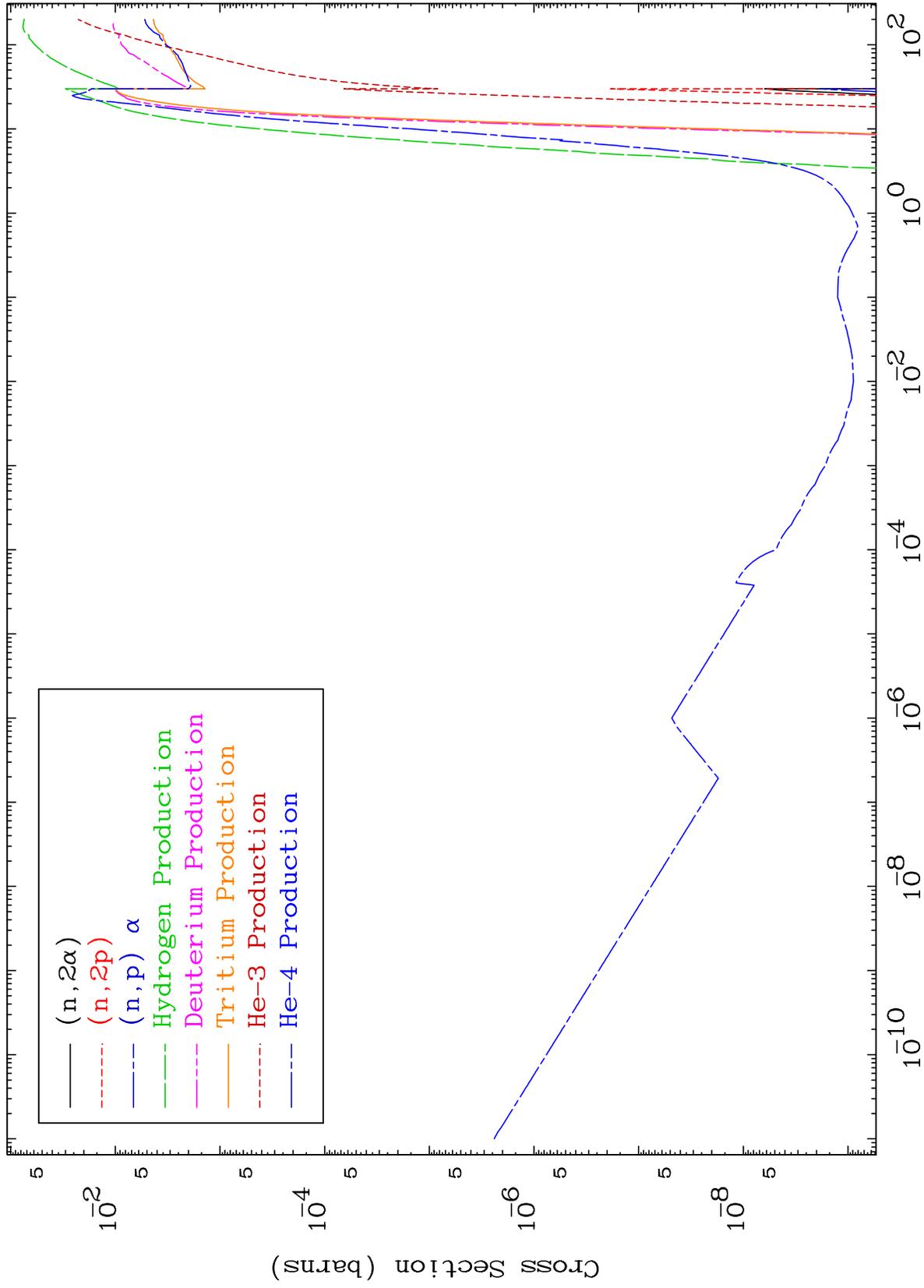
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MAT 9961

Neutron Absorption
293 Kelvin Cross Sections

101-Md-260



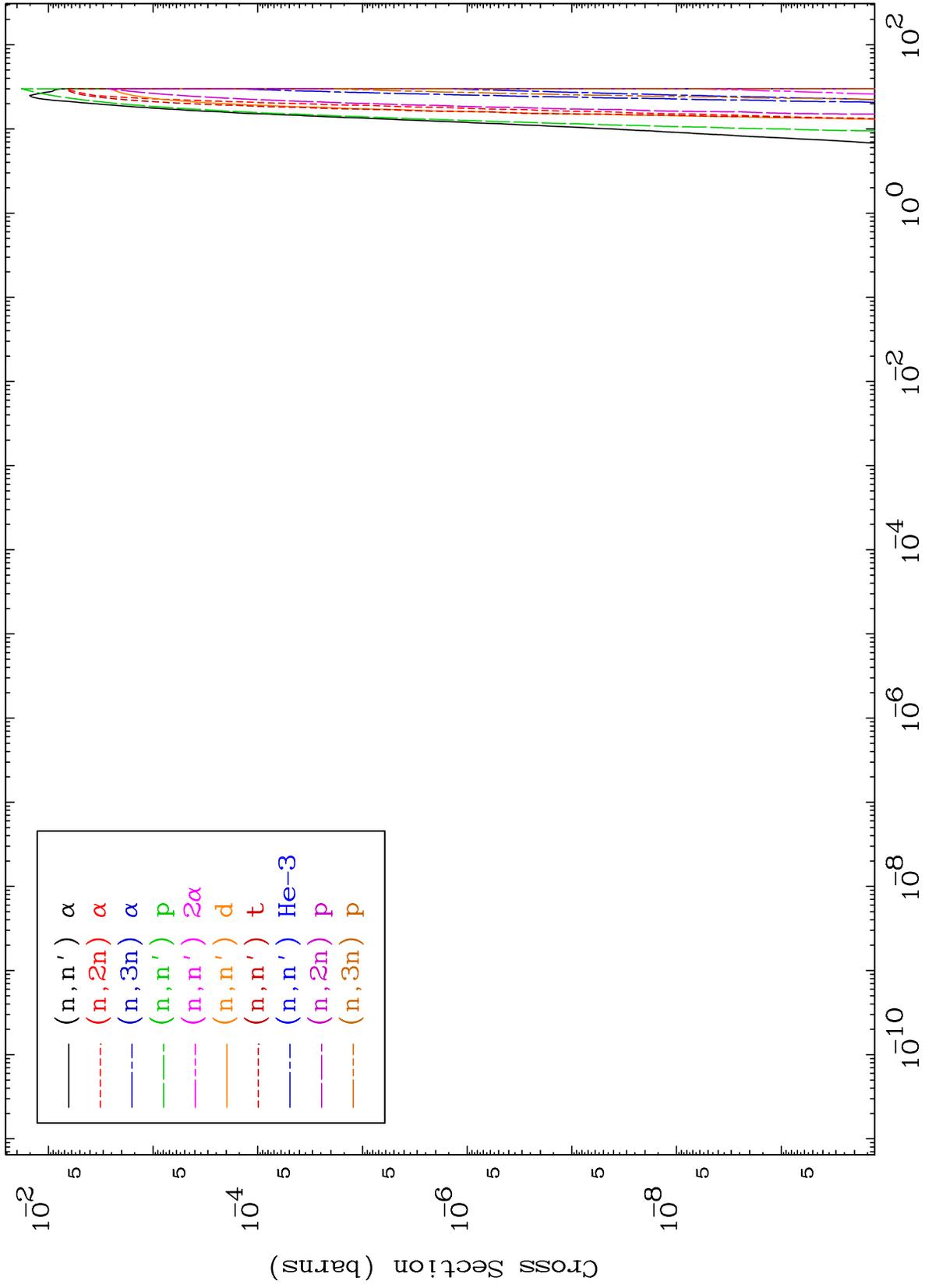
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101-Md-260

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293 Kelvin Cross Sections

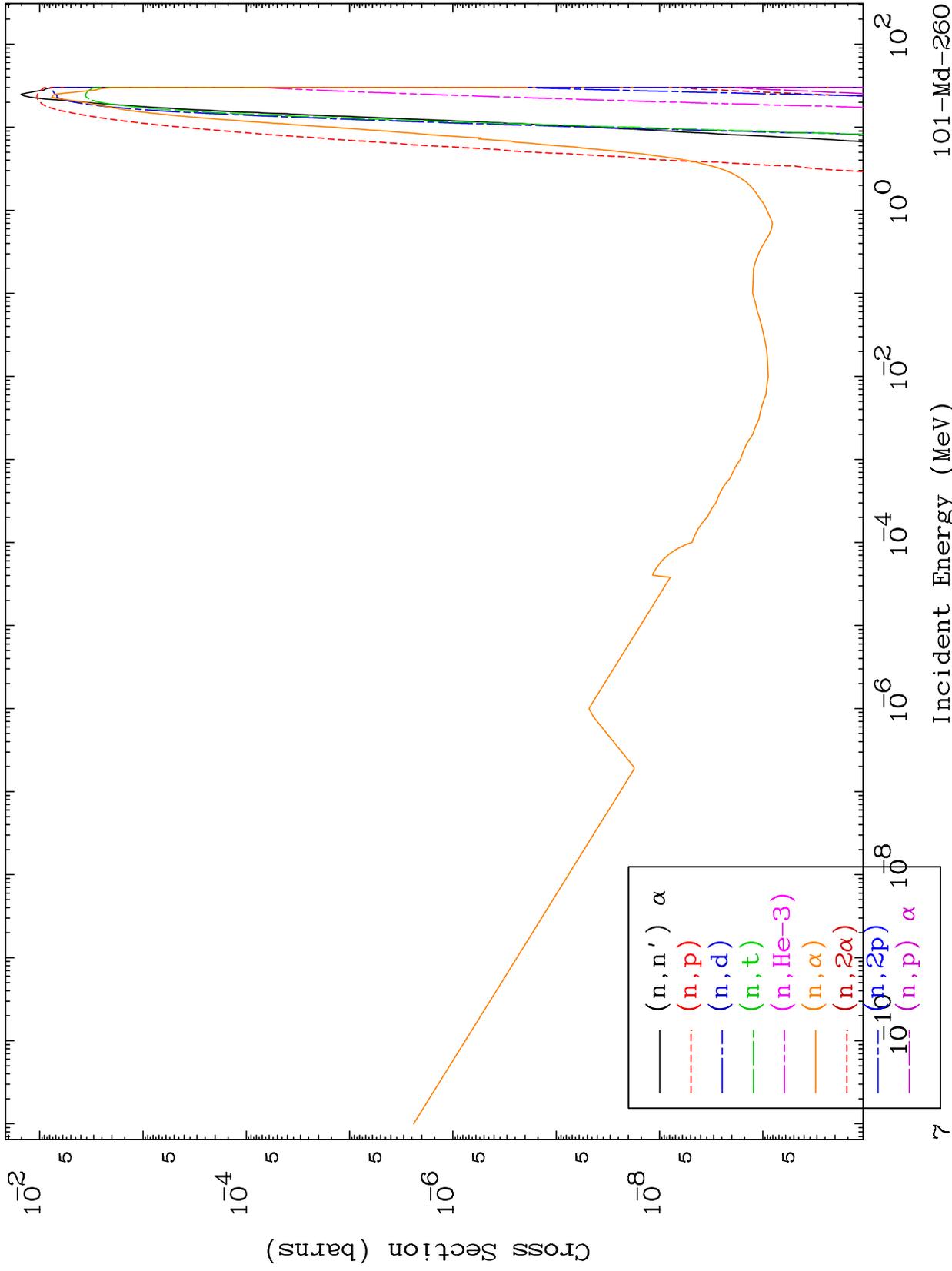
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MAT 9961

293 Kelvin Cross Sections

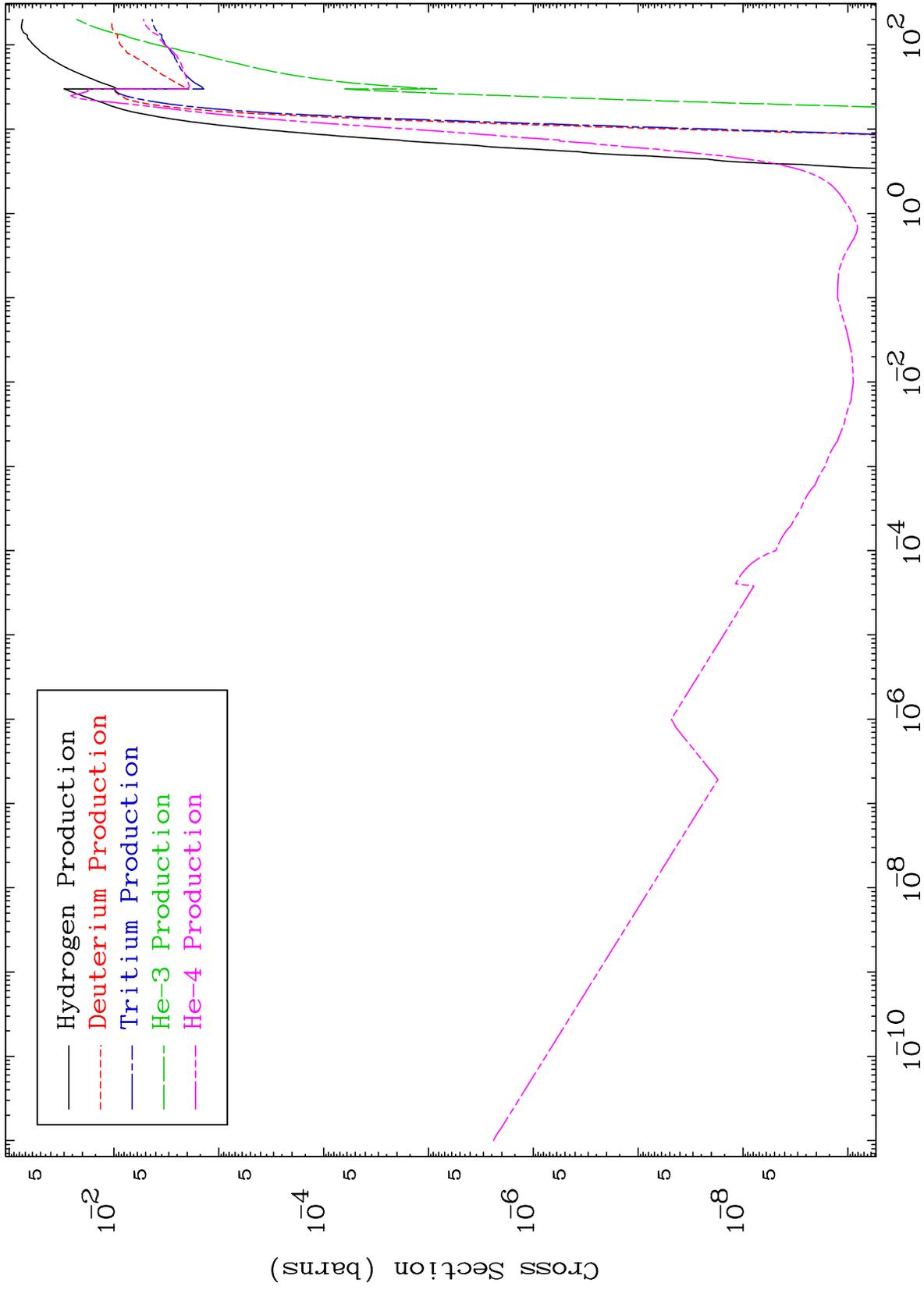
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Particle Production
293 Kelvin Cross Sections

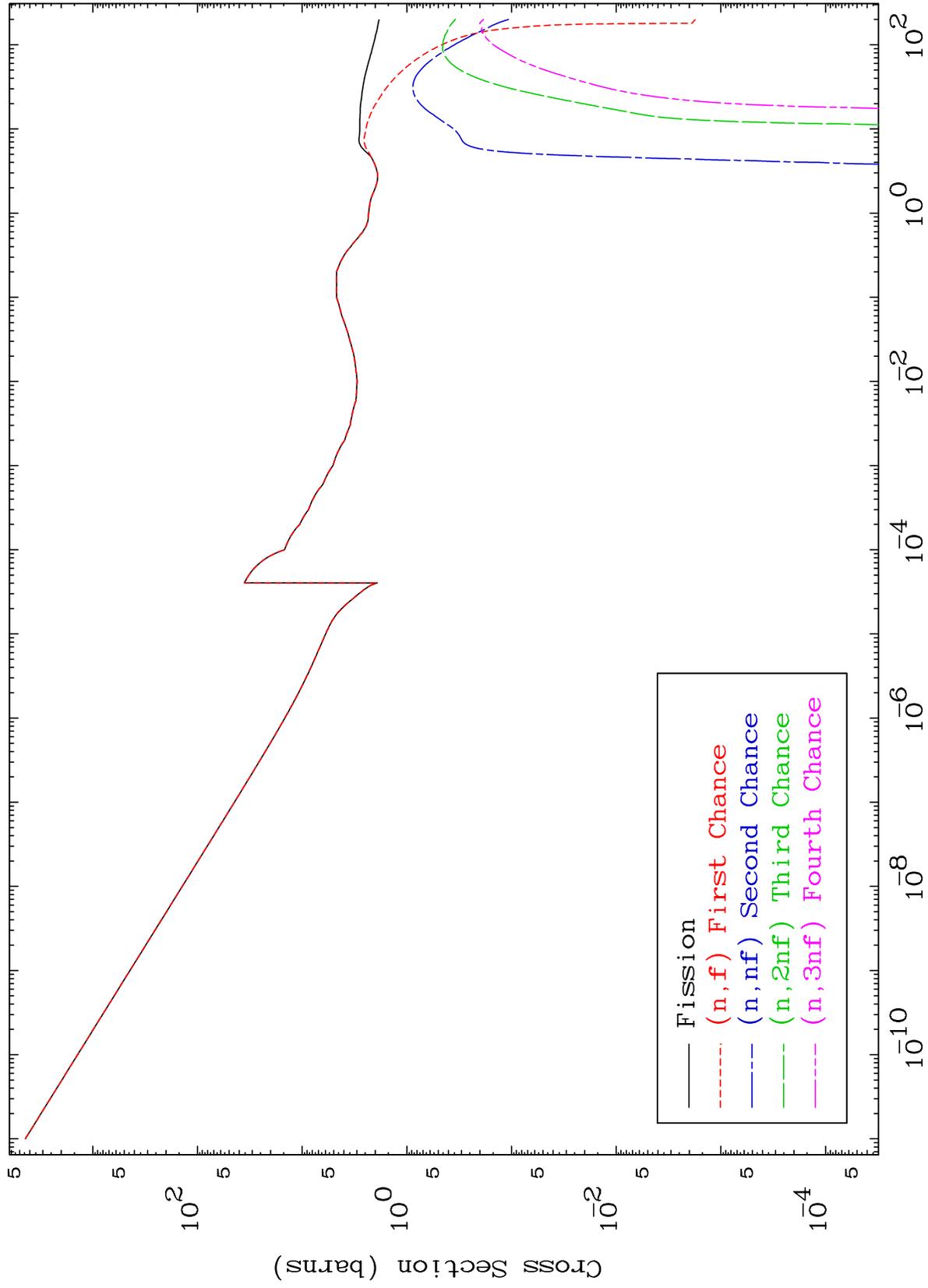
101-Md-260



MAT 9961

Fission
293 Kelvin Cross Sections

101-Md-260

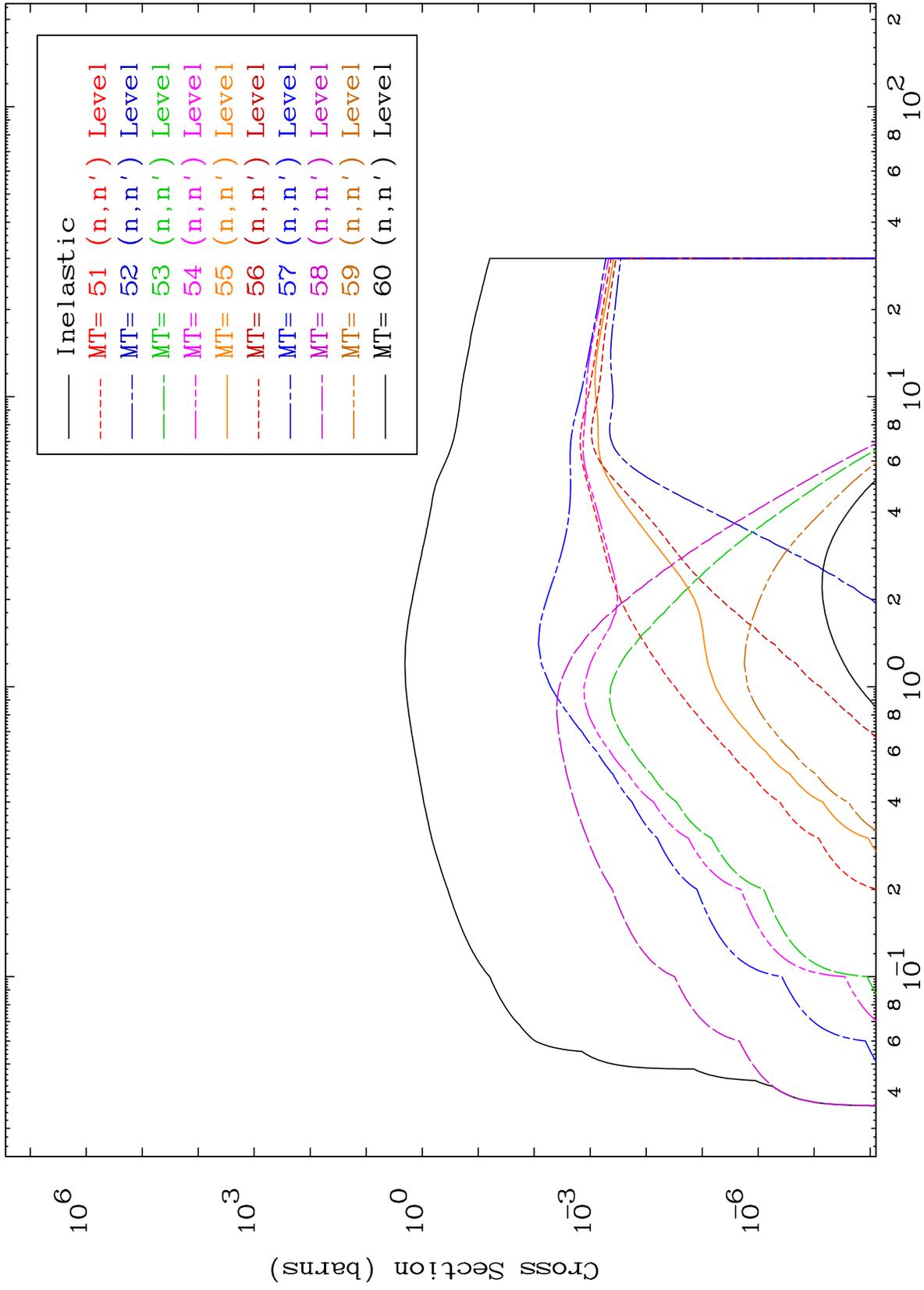


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(n,n') Levels

101-Md-260

293 Kelvin Cross Sections



10

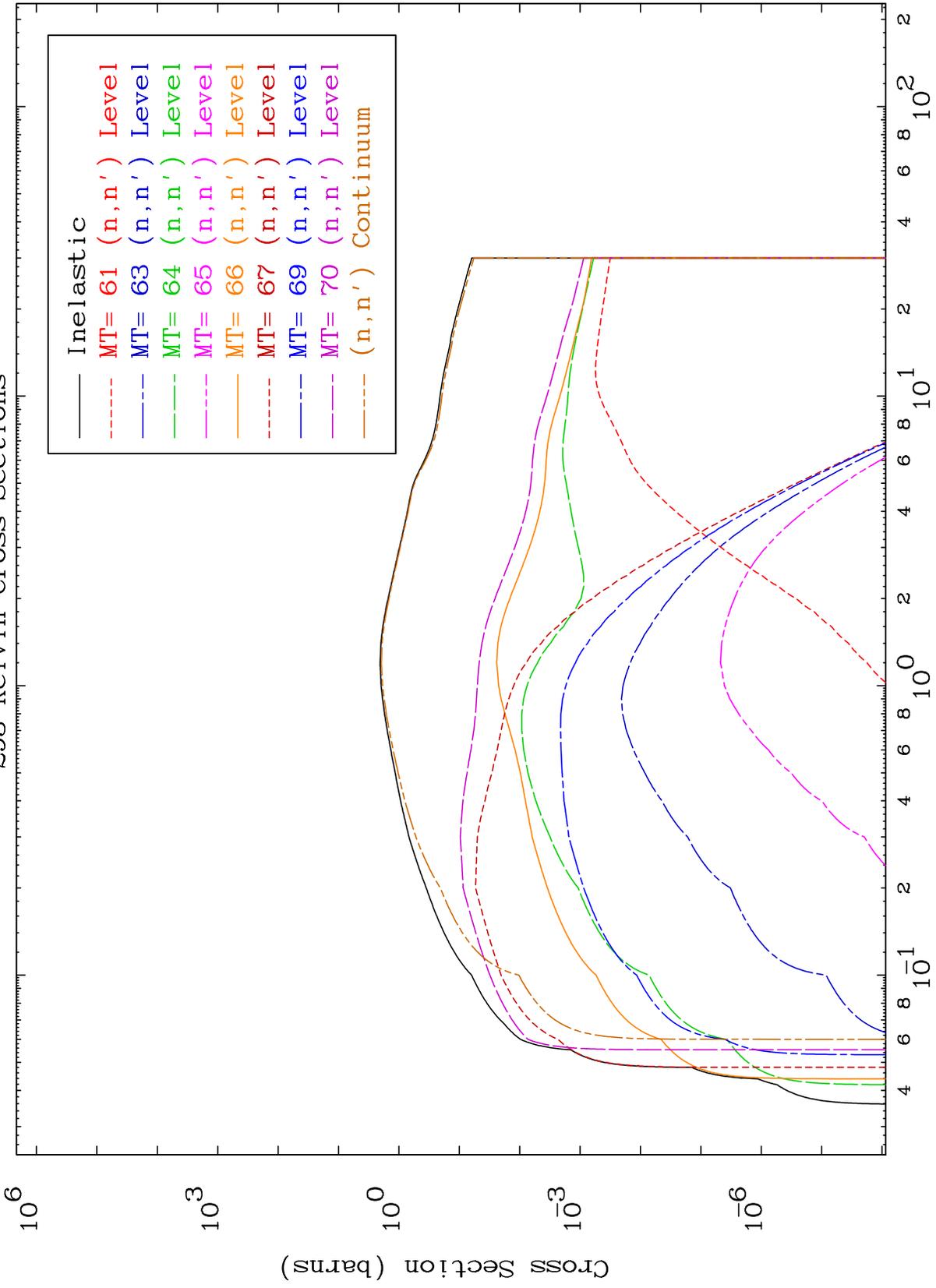
Incident Energy (MeV)

101-Md-260

MAT 9961

(n,n') Levels
293 Kelvin Cross Sections

101-Md-260



11

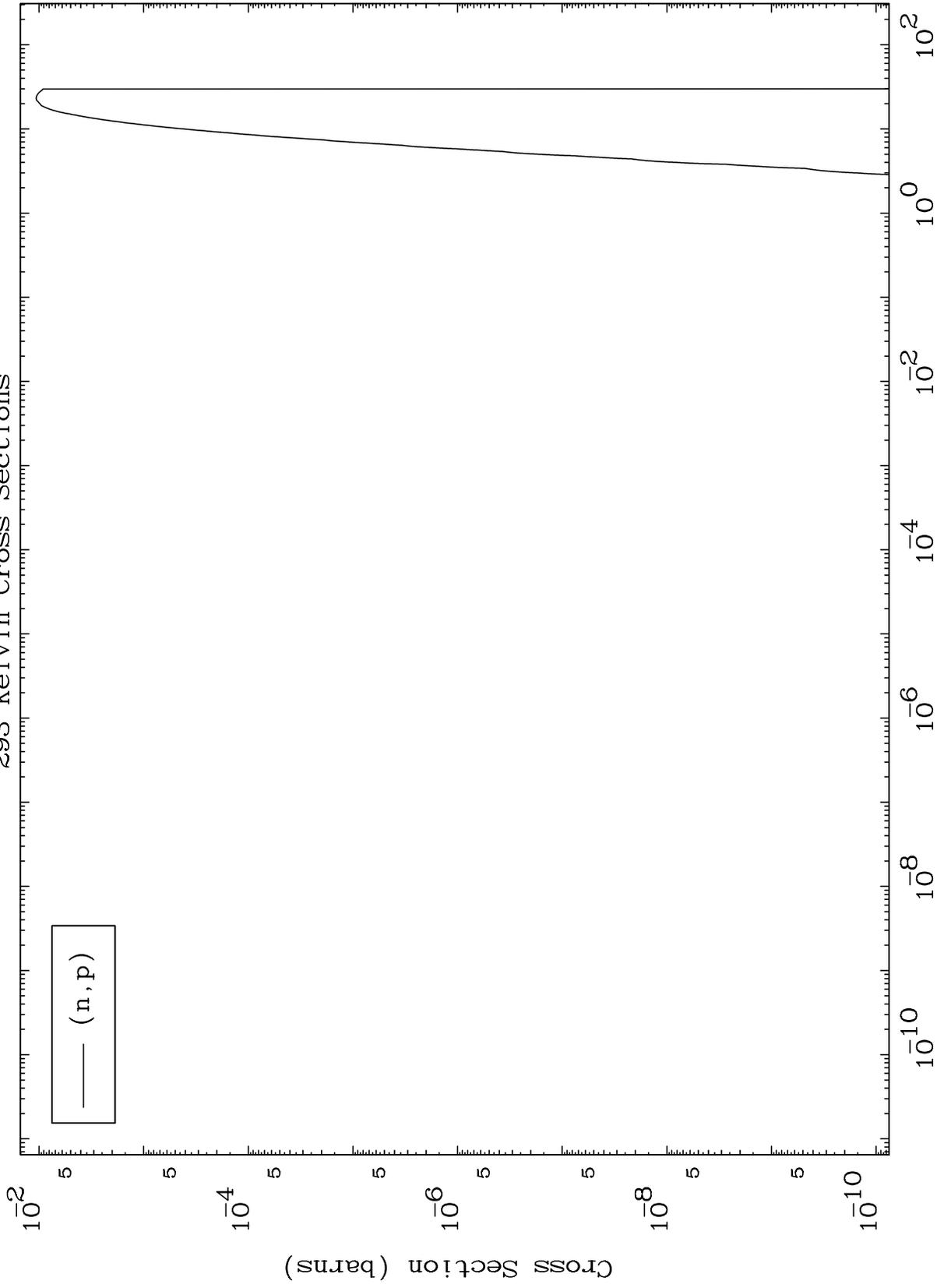
Incident Energy (MeV)

101-Md-260

MAT 9961

(n,p) Levels
293 Kelvin Cross Sections

101-Md-260



12

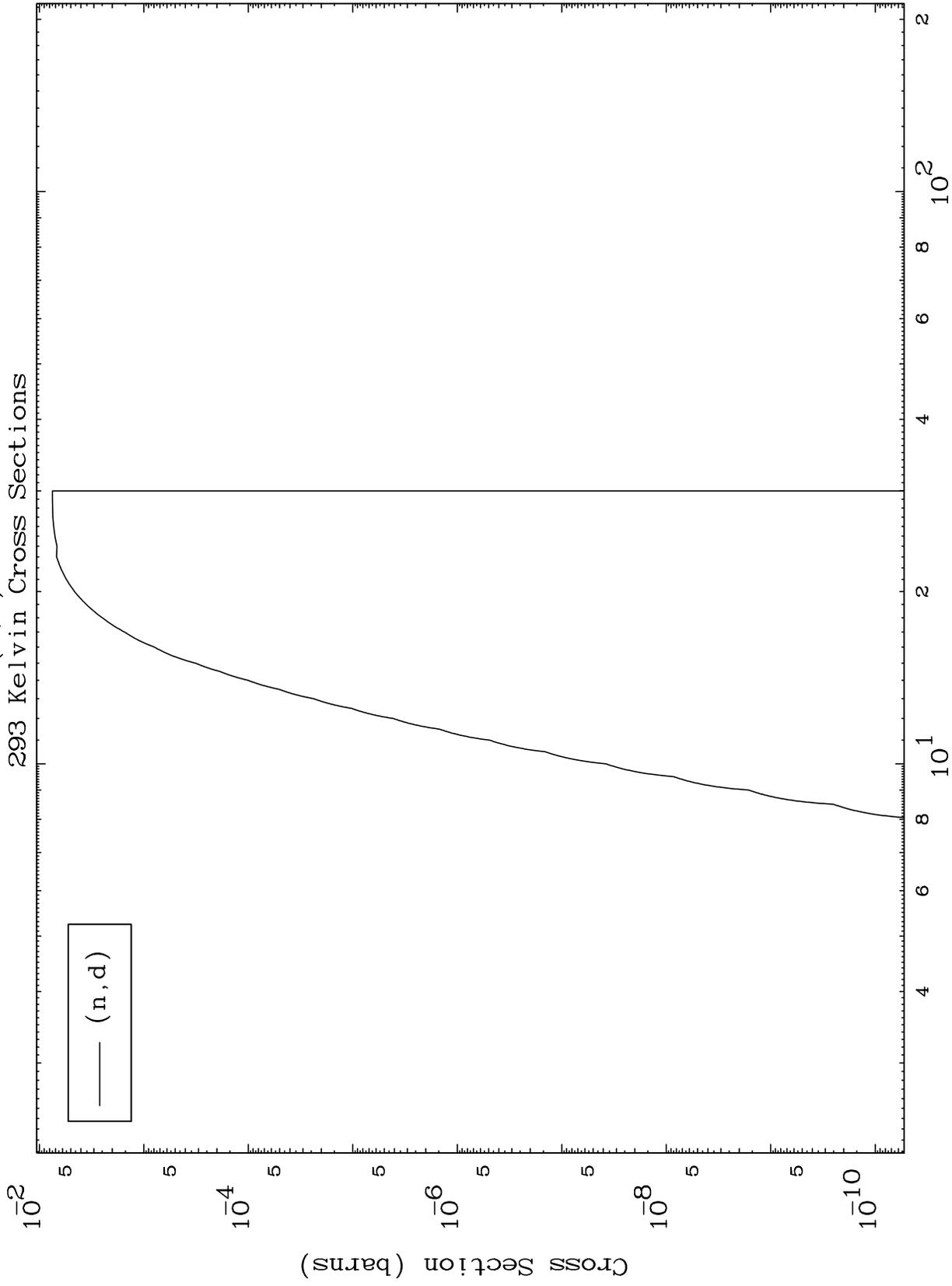
Incident Energy (MeV)

101-Md-260

MAT 9961

(n,d) Levels
293 Kelvin Cross Sections

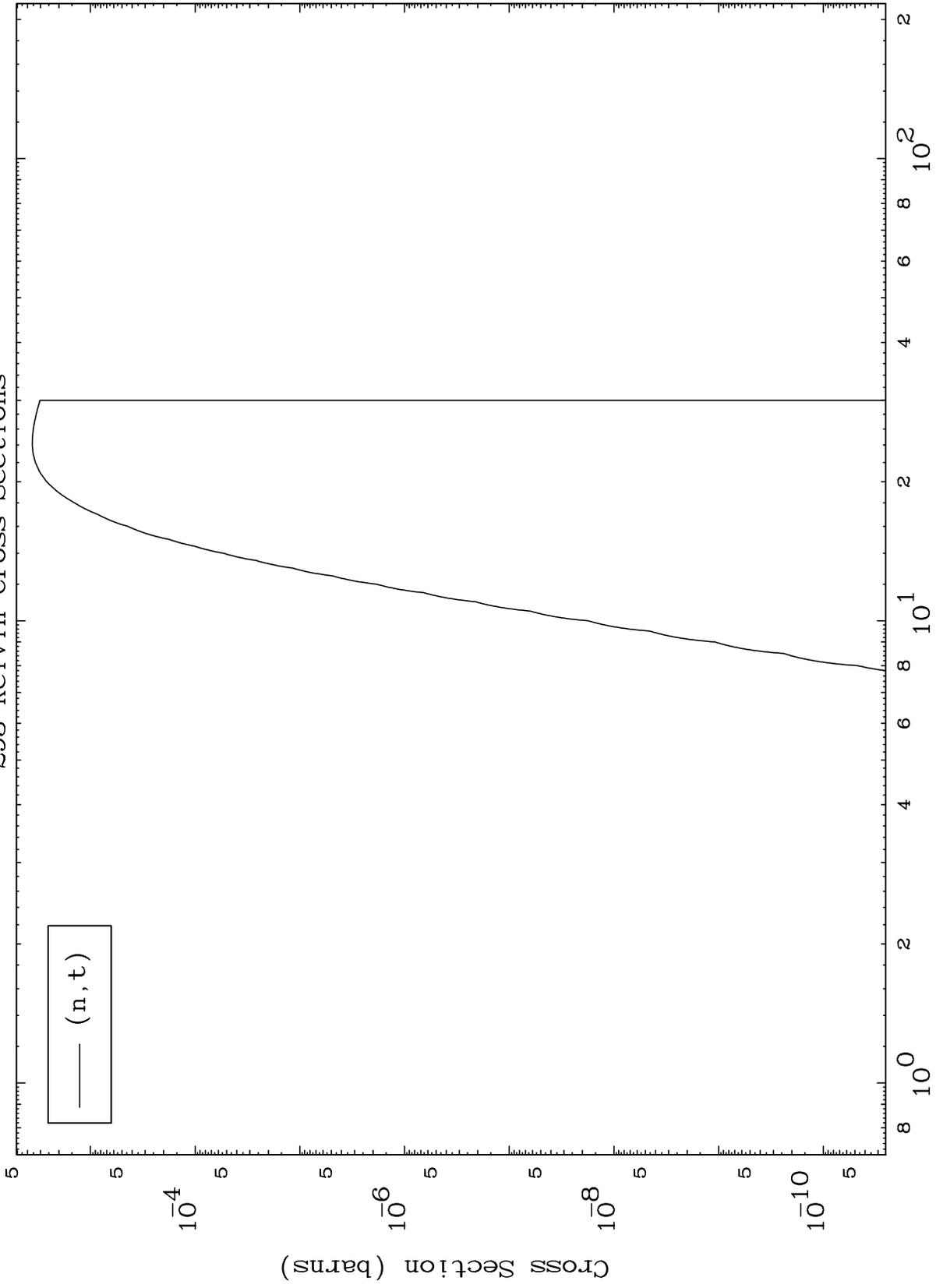
101-Md-260



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(n,t) Levels
293 Kelvin Cross Sections

101-Md-260



14

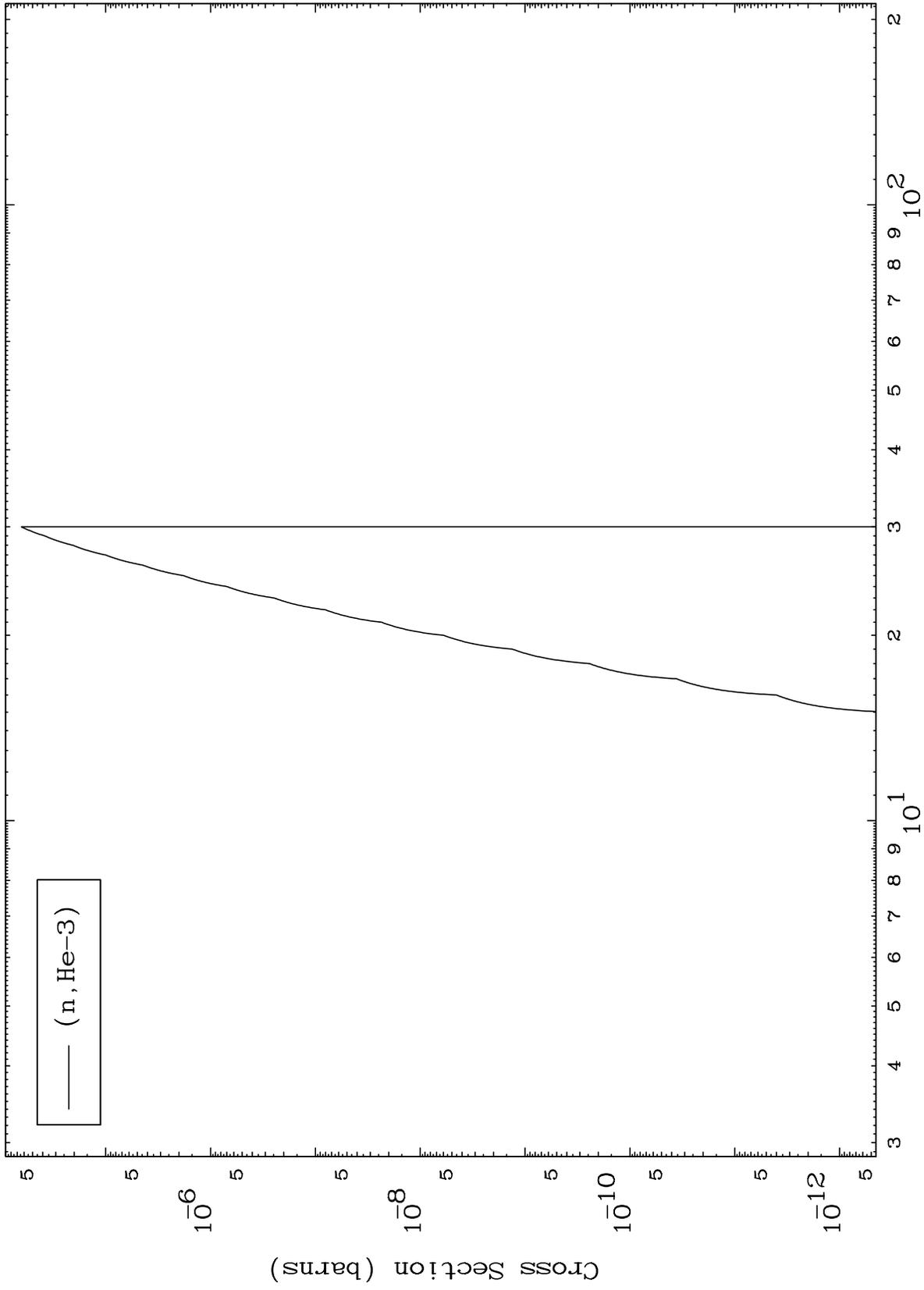
Incident Energy (MeV)

101-Md-260

MAT 9961

(n,He3) Levels
293 Kelvin Cross Sections

101-Md-260



15

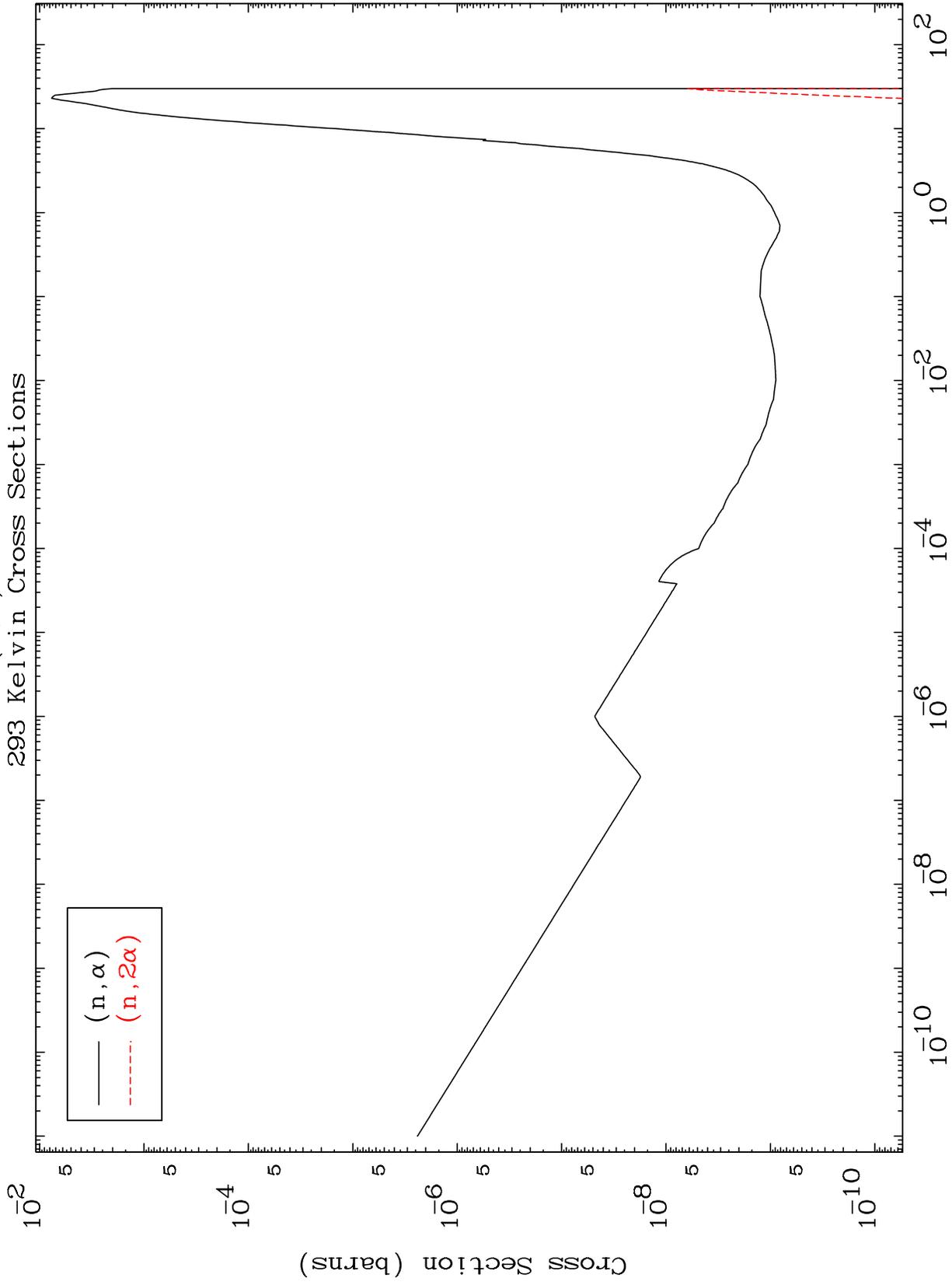
Incident Energy (MeV)

101-Md-260

MAT 9961

(n, α) Levels
293 Kelvin Cross Sections

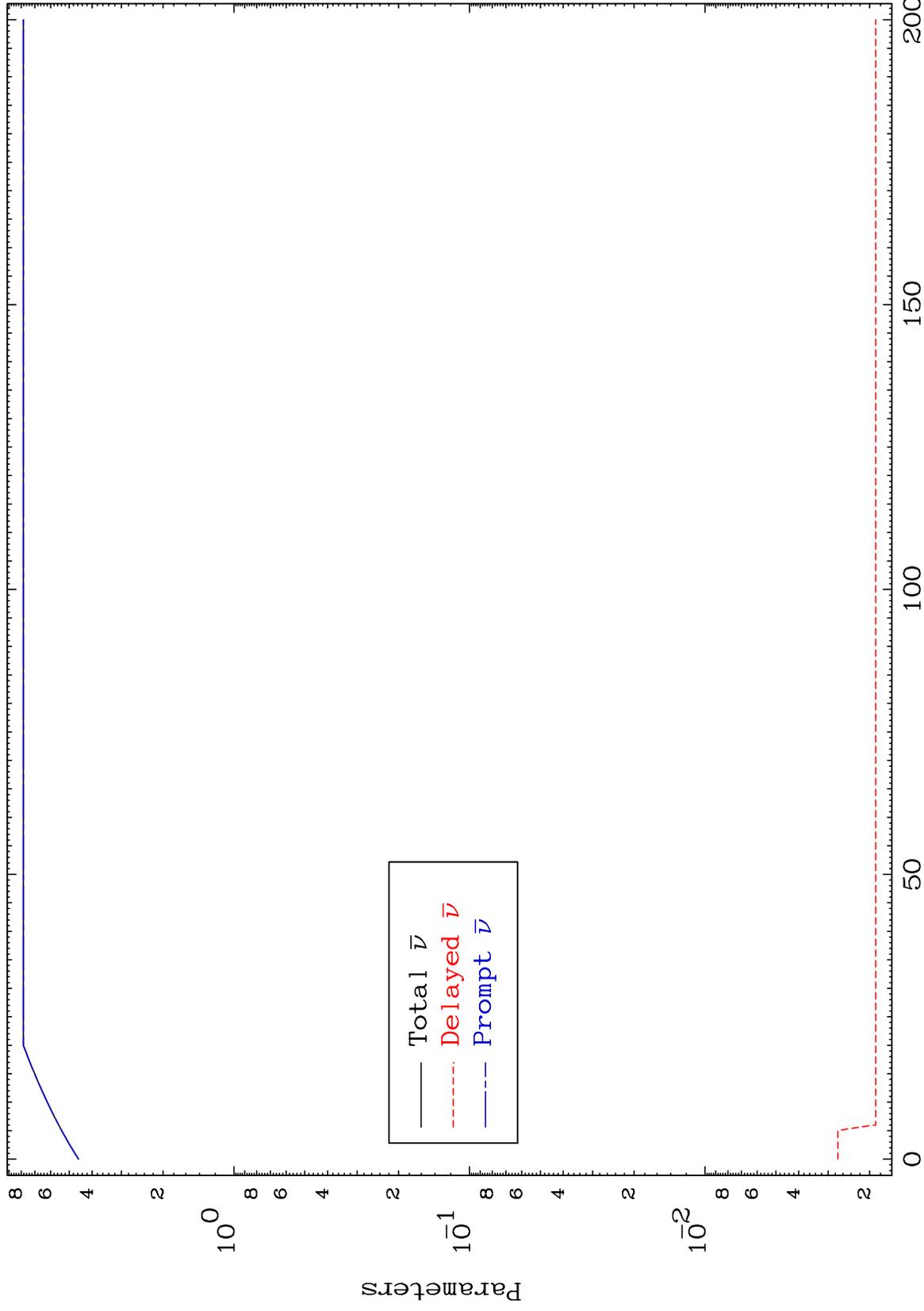
101-Md-260



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Energy Release
Parameters

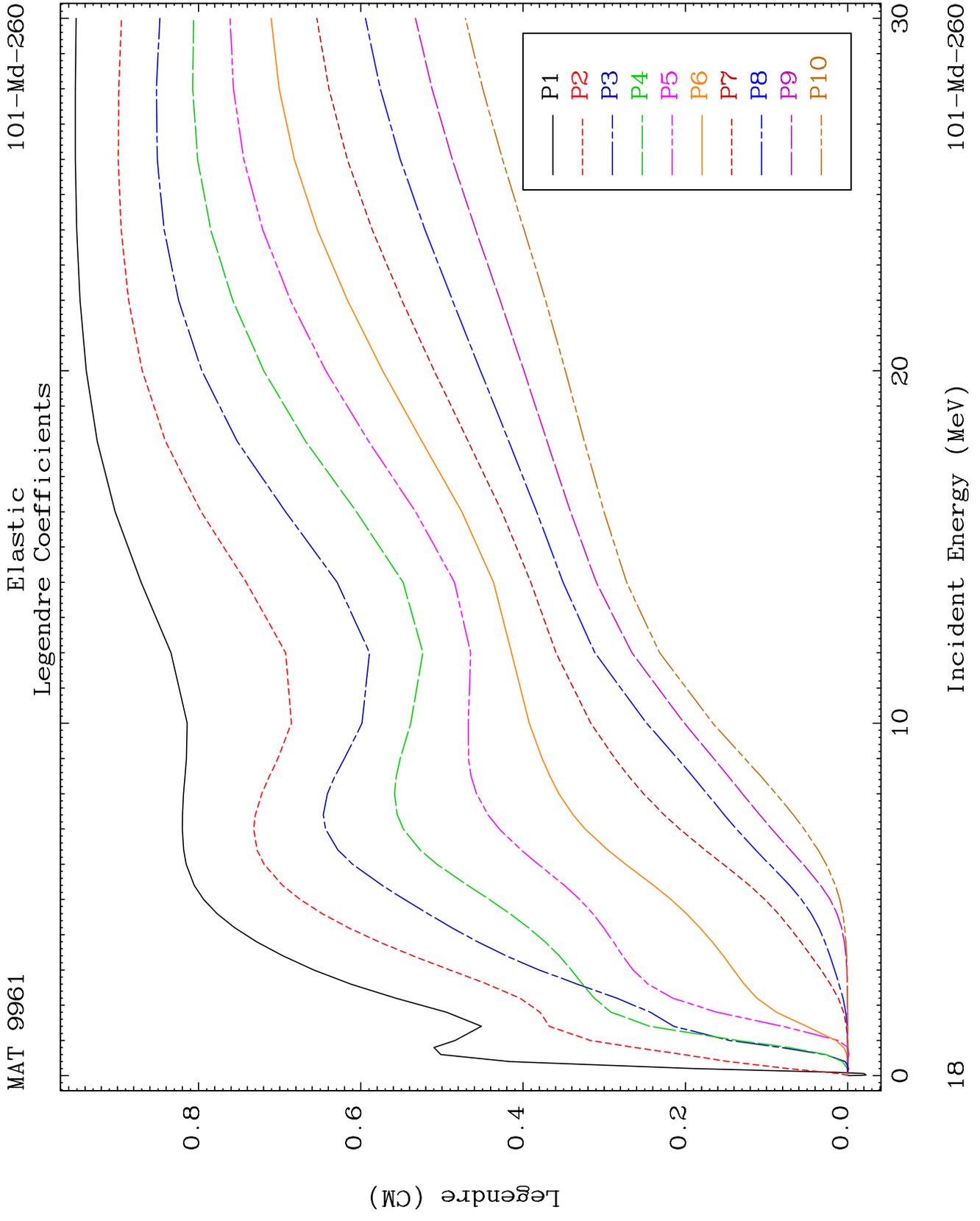
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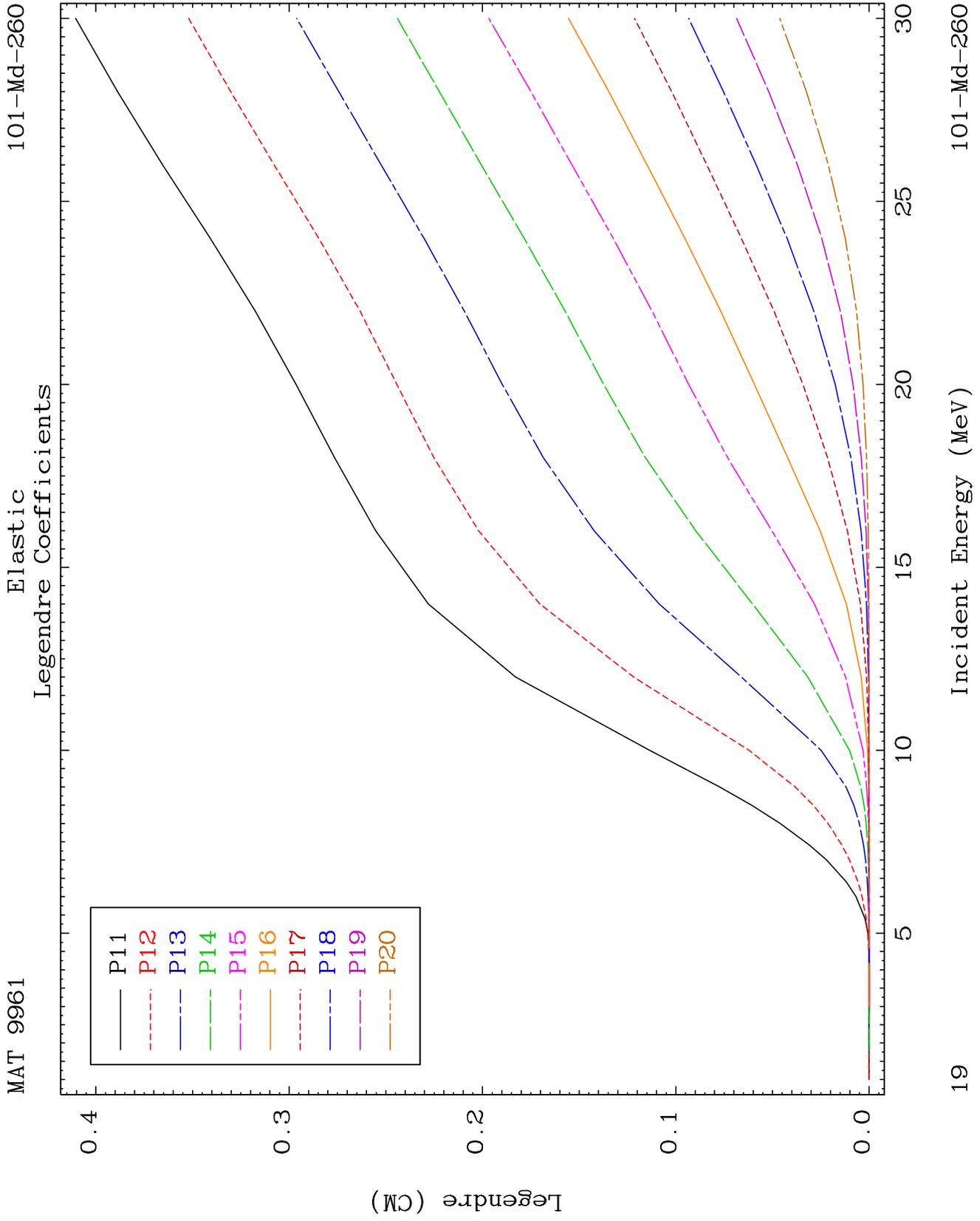


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Incident Energy (MeV)

101-Md-260

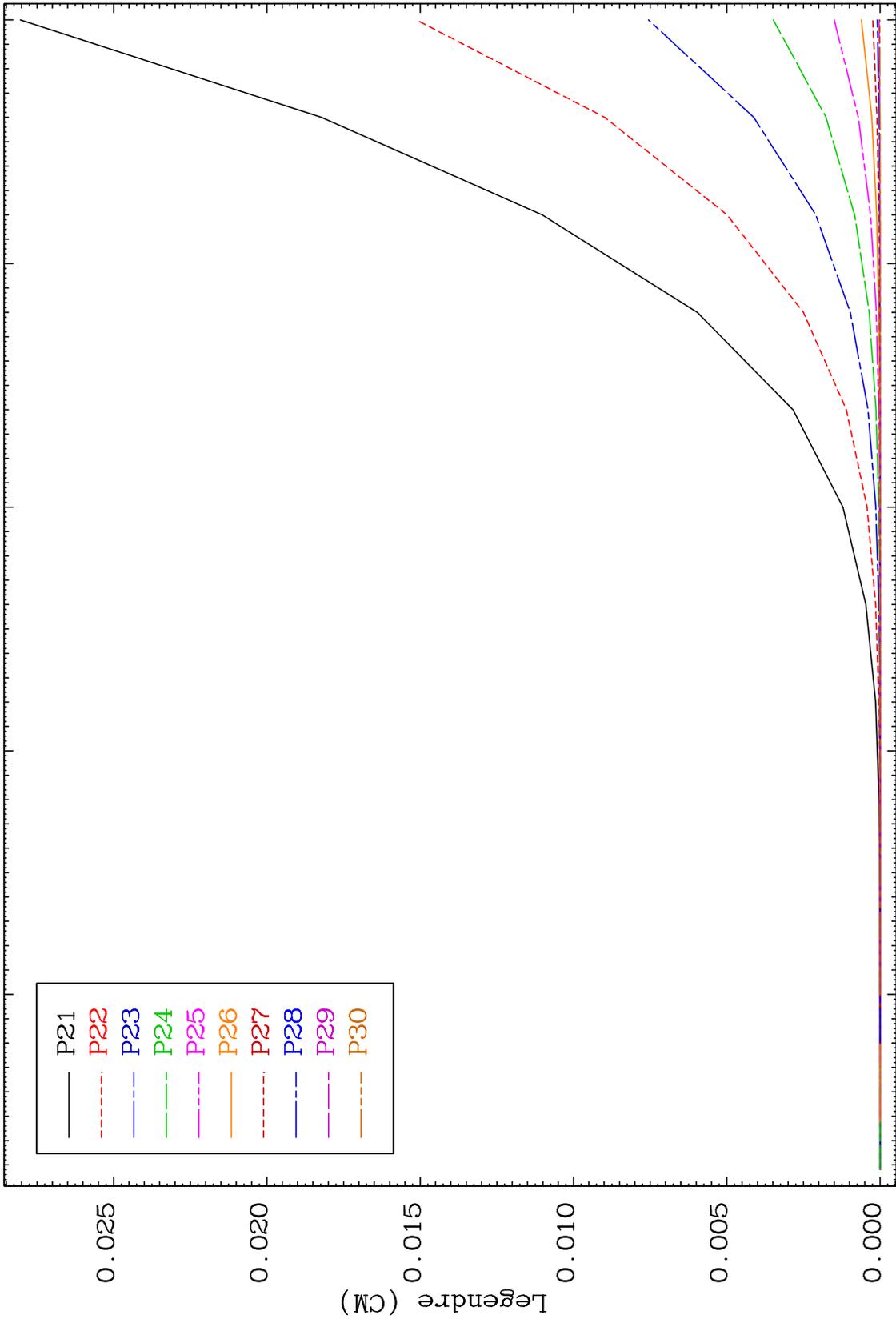




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Elastic Legendre Coefficients

101-Md-260



20

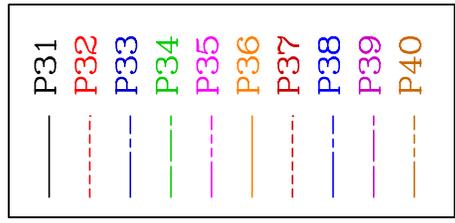
Incident Energy (MeV)

101-Md-260

MAT 9961

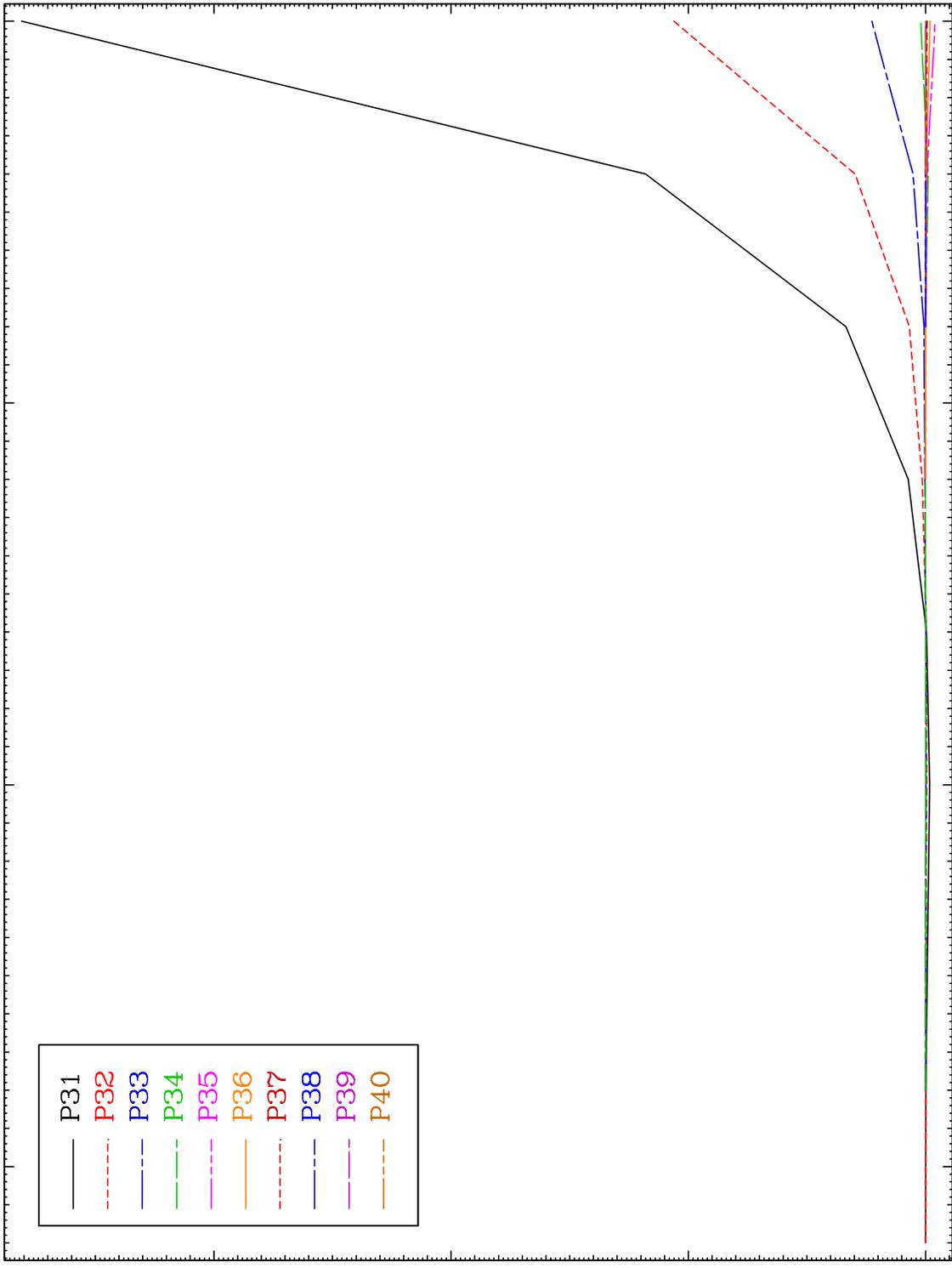
Elastic Legendre Coefficients

101-Md-260



$\times 10^{-6}$

Legendre (CM)



21

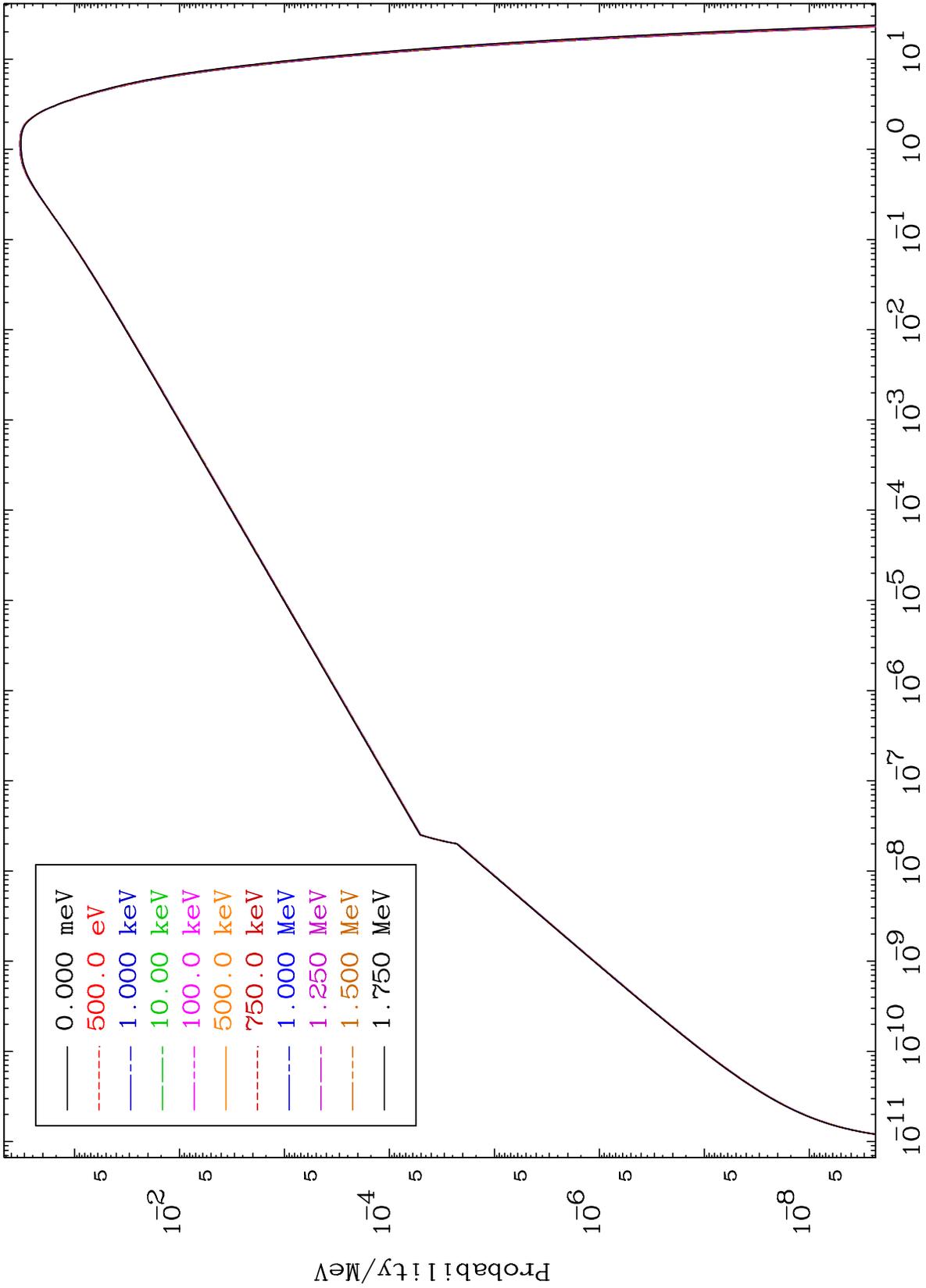
Incident Energy (MeV)

101-Md-260

MAT 9961

Fission Energy Distributions

101-Md-260

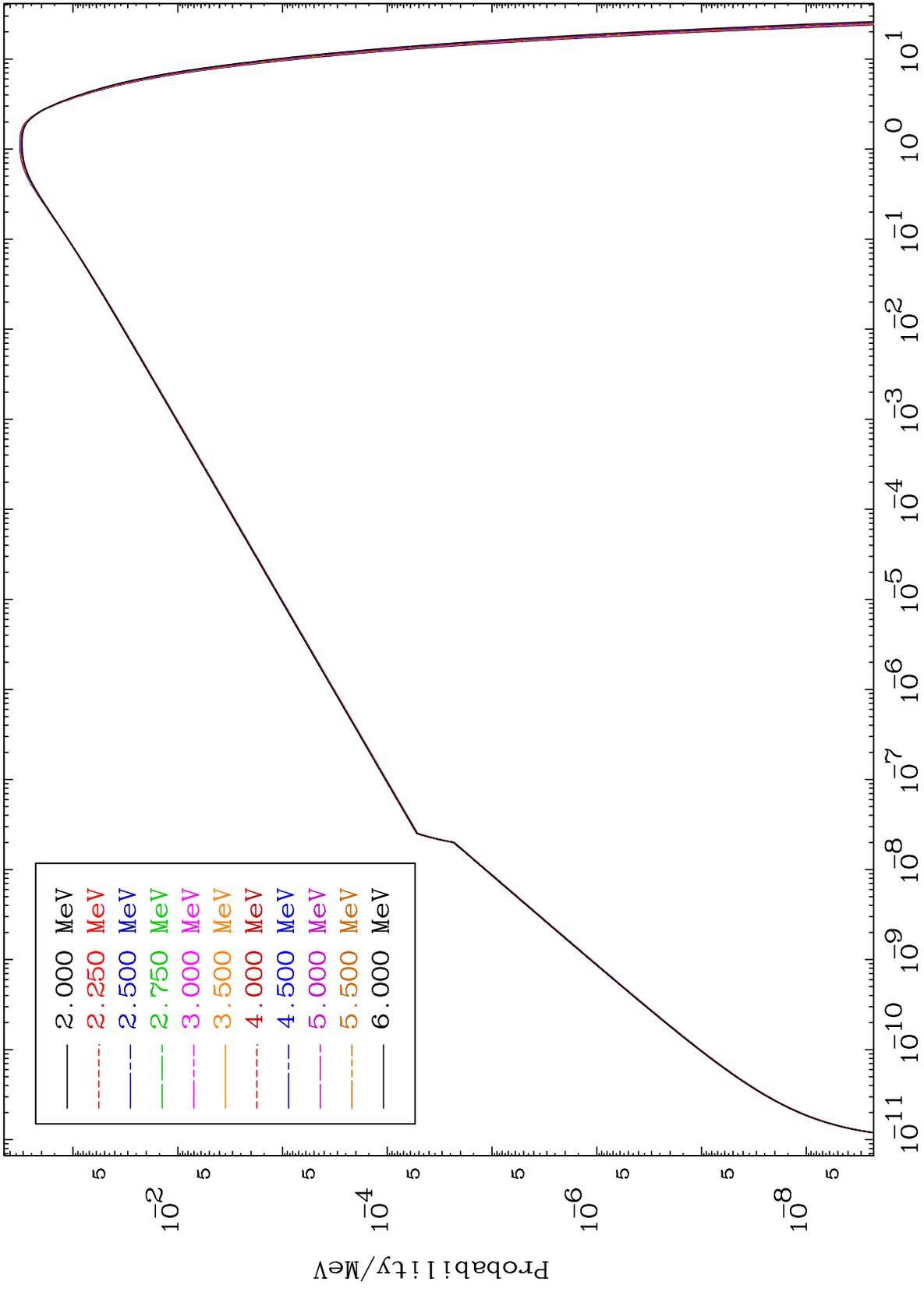


101-Md-260

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Fission Energy Distributions

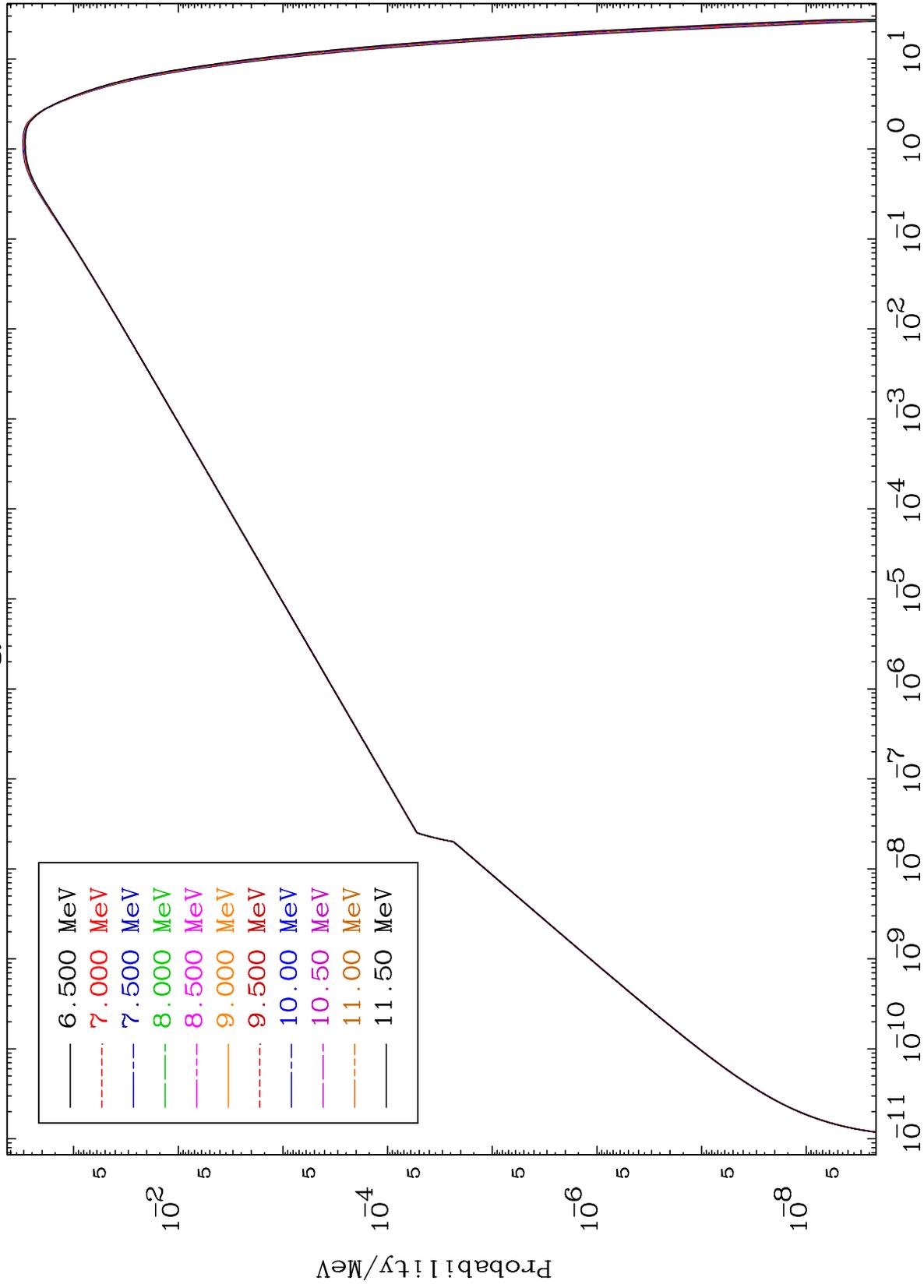
101-Md-260



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Fission Energy Distributions

101-Md-260

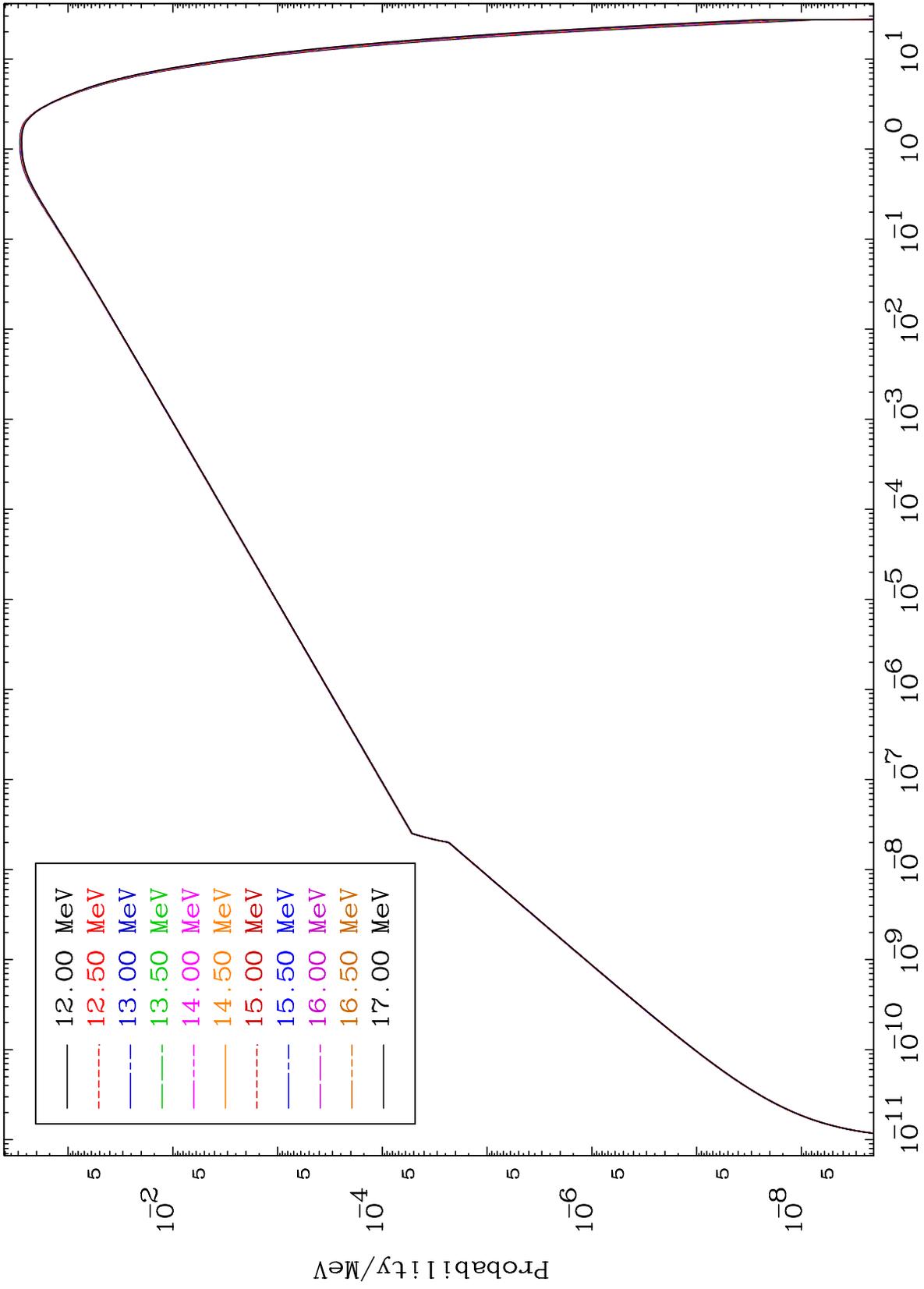


101-Md-260

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Fission Energy Distributions

101-Md-260

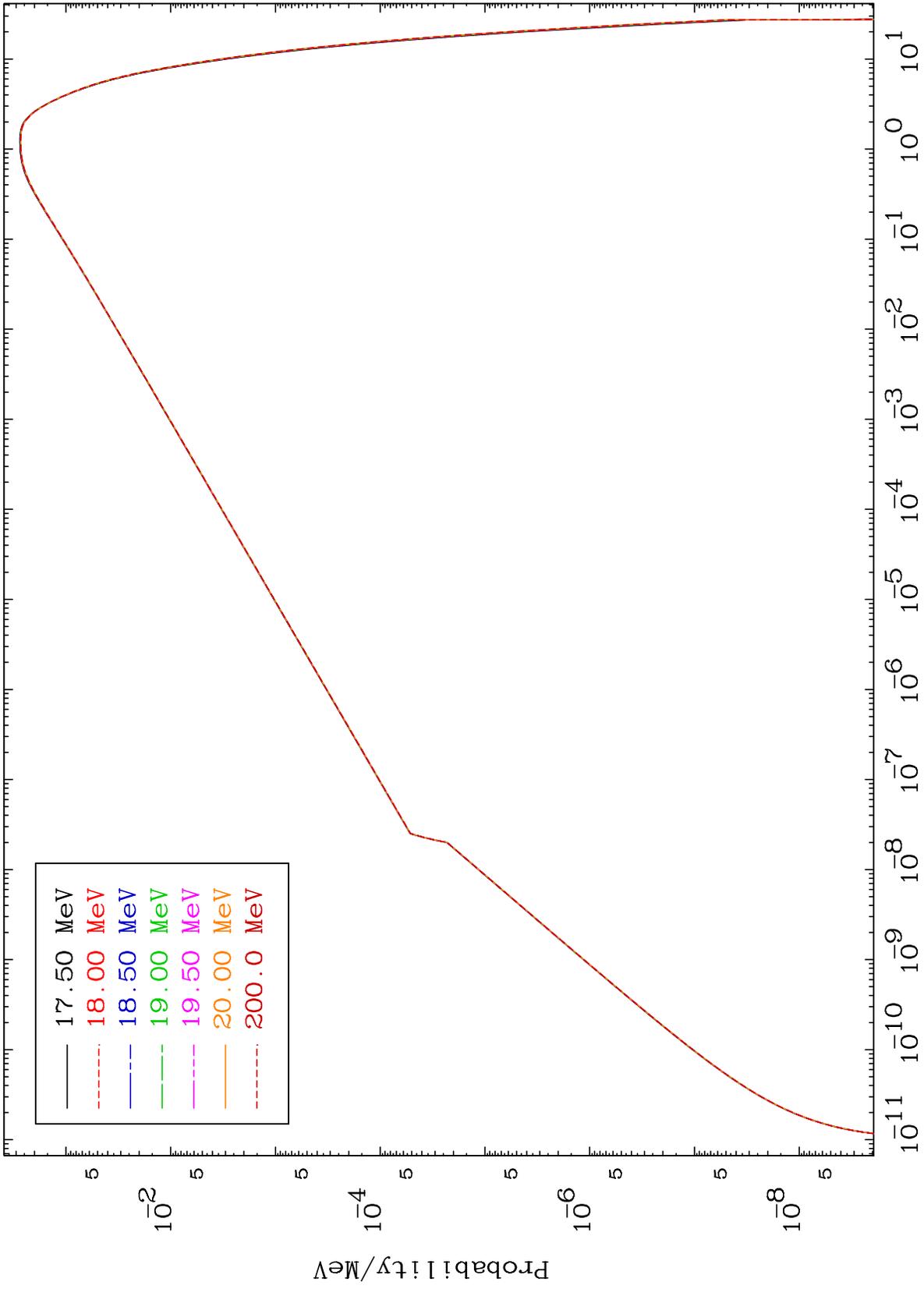


101-Md-260

MAT 9961

Fission Energy Distributions

101-Md-260



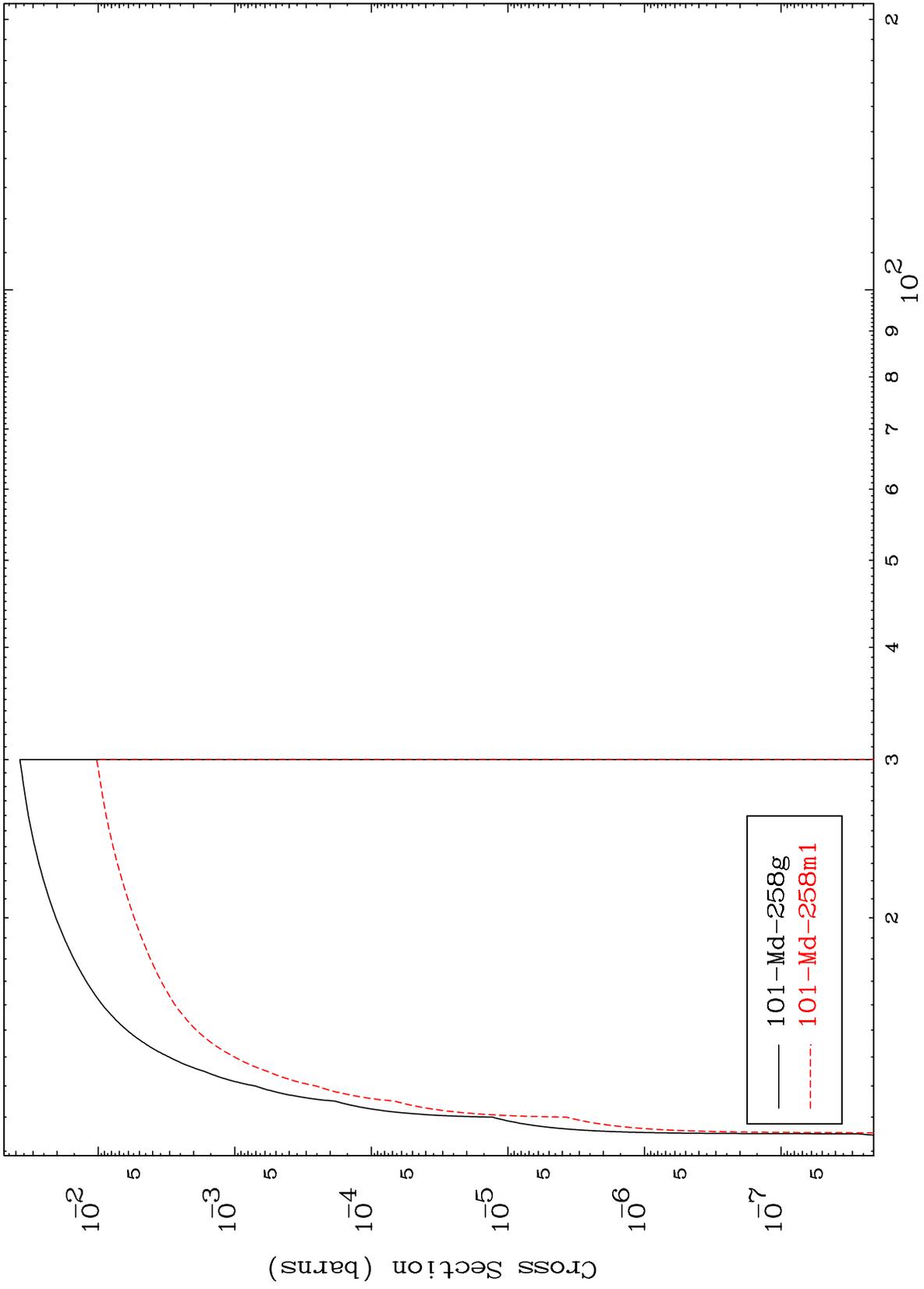
—	17.50 MeV
- - -	18.00 MeV
- . - .	18.50 MeV
- - -	19.00 MeV
- . - .	19.50 MeV
- - -	20.00 MeV
- - -	200.0 MeV

101-Md-260

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101-Md-260

(n,3n)
Radionuclide Production Cross Section



27

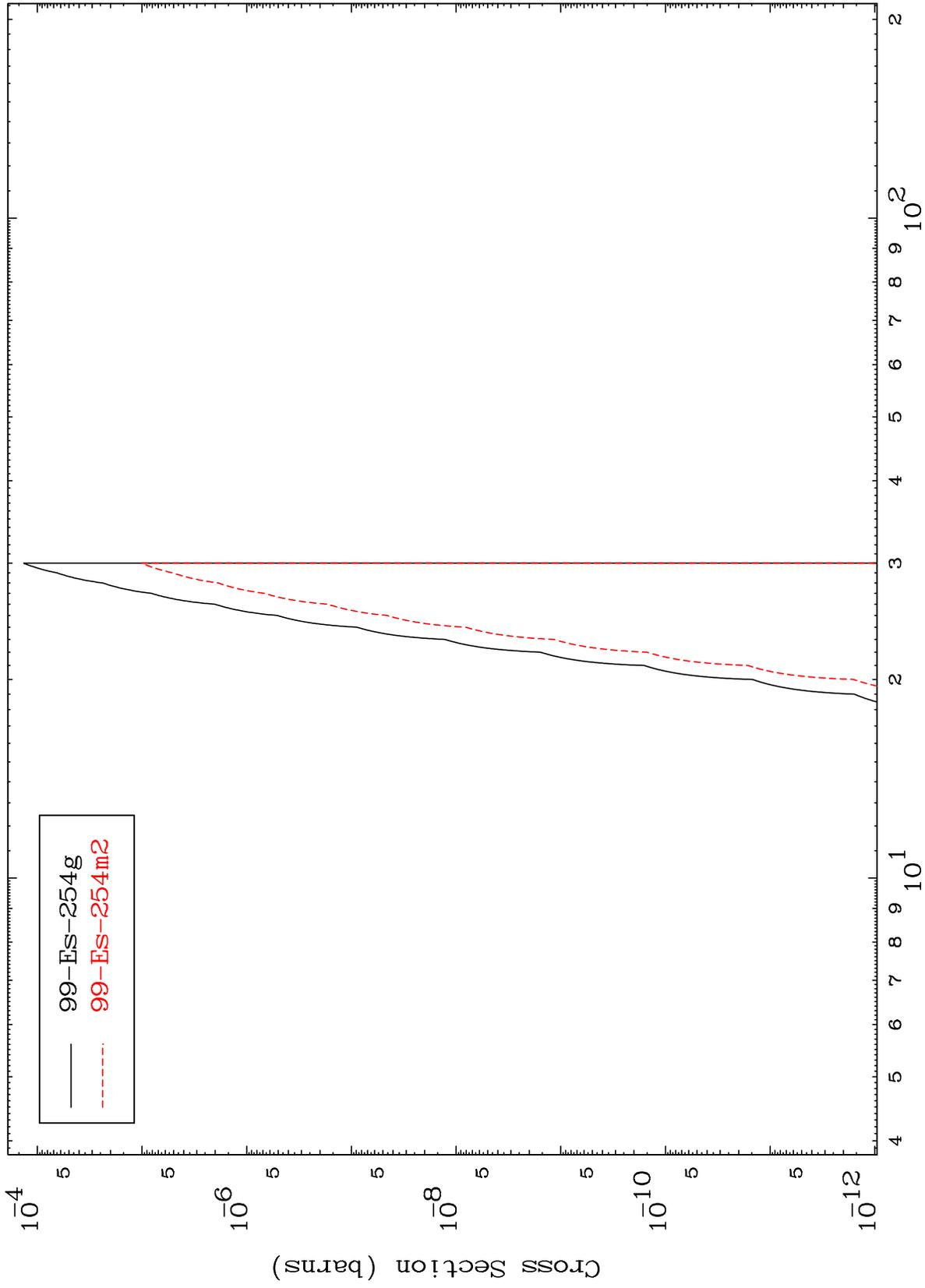
101-Md-260

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(n,3n) α

101-Md-260

Radionuclide Production Cross Section



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Incident Energy (MeV)

101-Md-260