

Program EVALPLOT  
(Version 2021-1)

by

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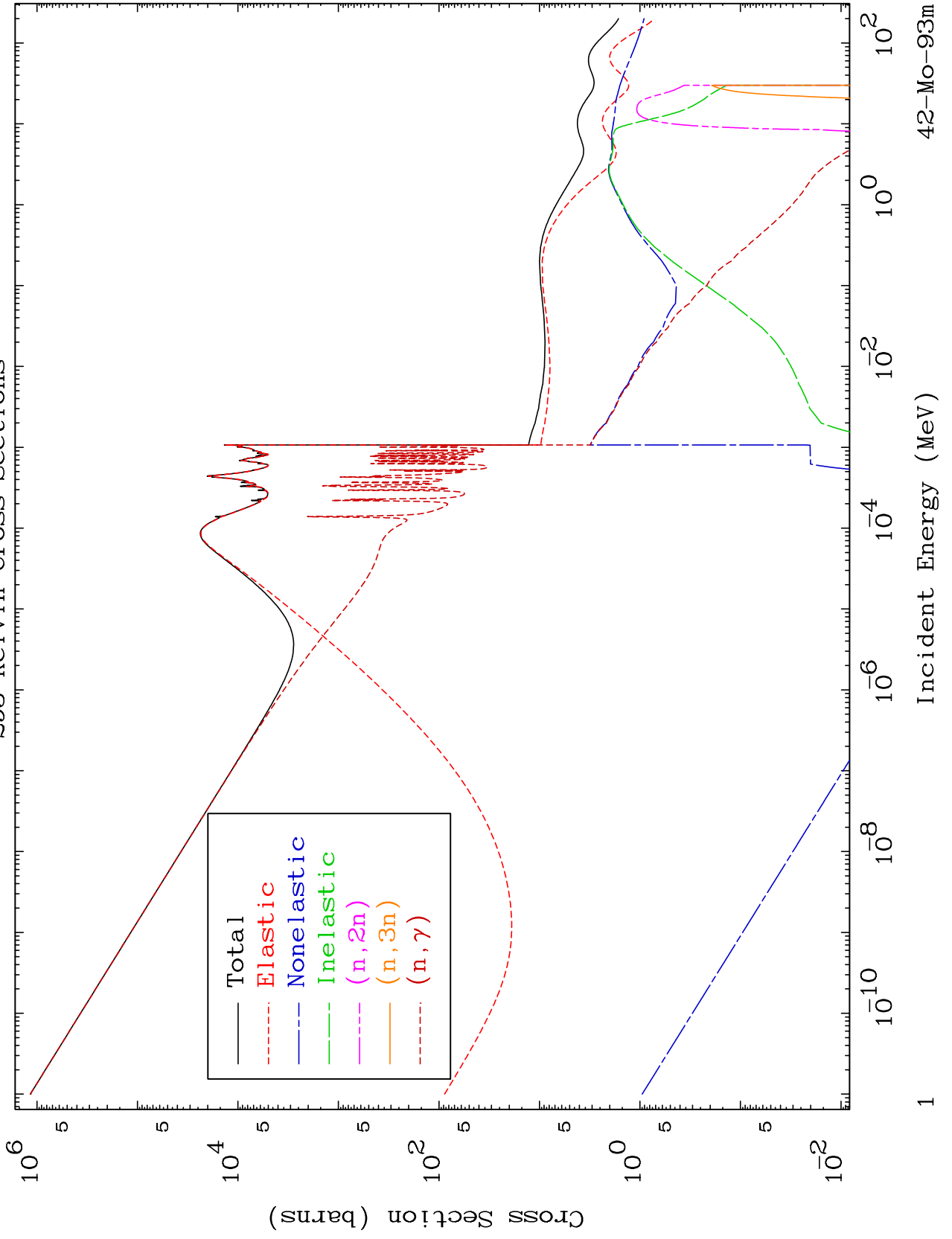
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 4229

Neutron Major  
293 Kelvin Cross Sections

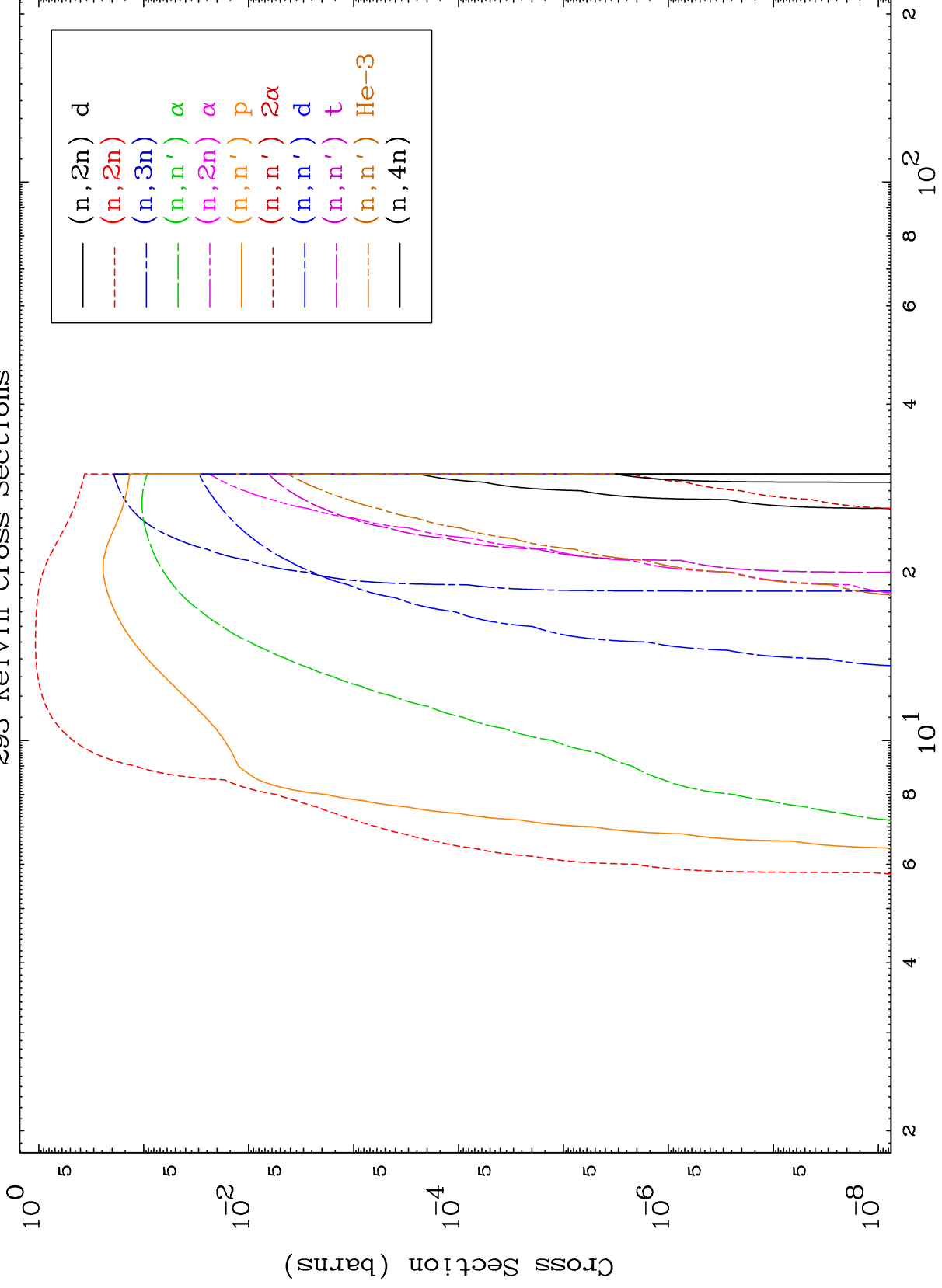
42-Mo-93m



MAT 4229

Neutron Absorption  
293 Kelvin Cross Sections

42-Mo-93m



2

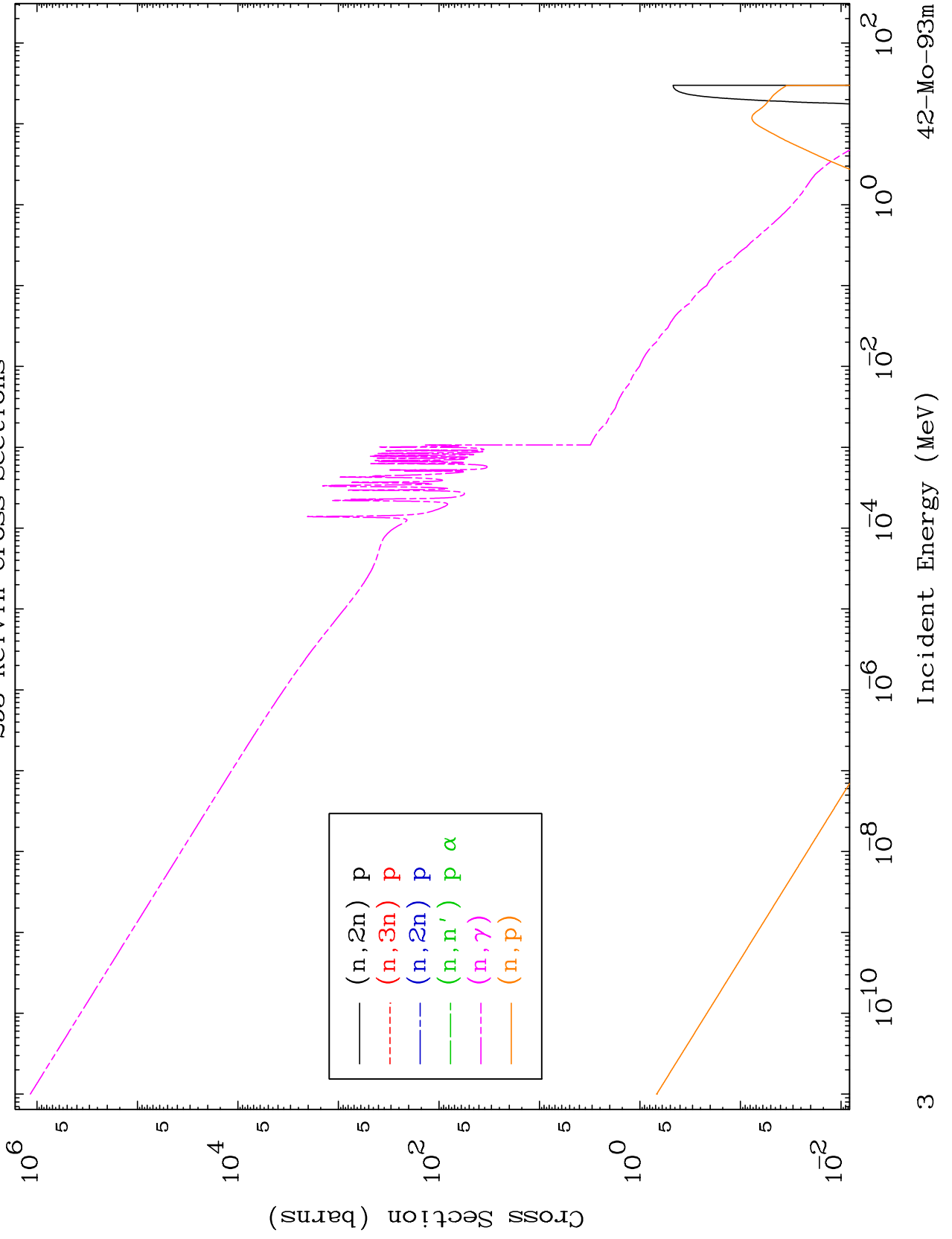
Incident Energy (MeV)

42-Mo-93m

MAT 4229

Neutron Absorption  
293 Kelvin Cross Sections

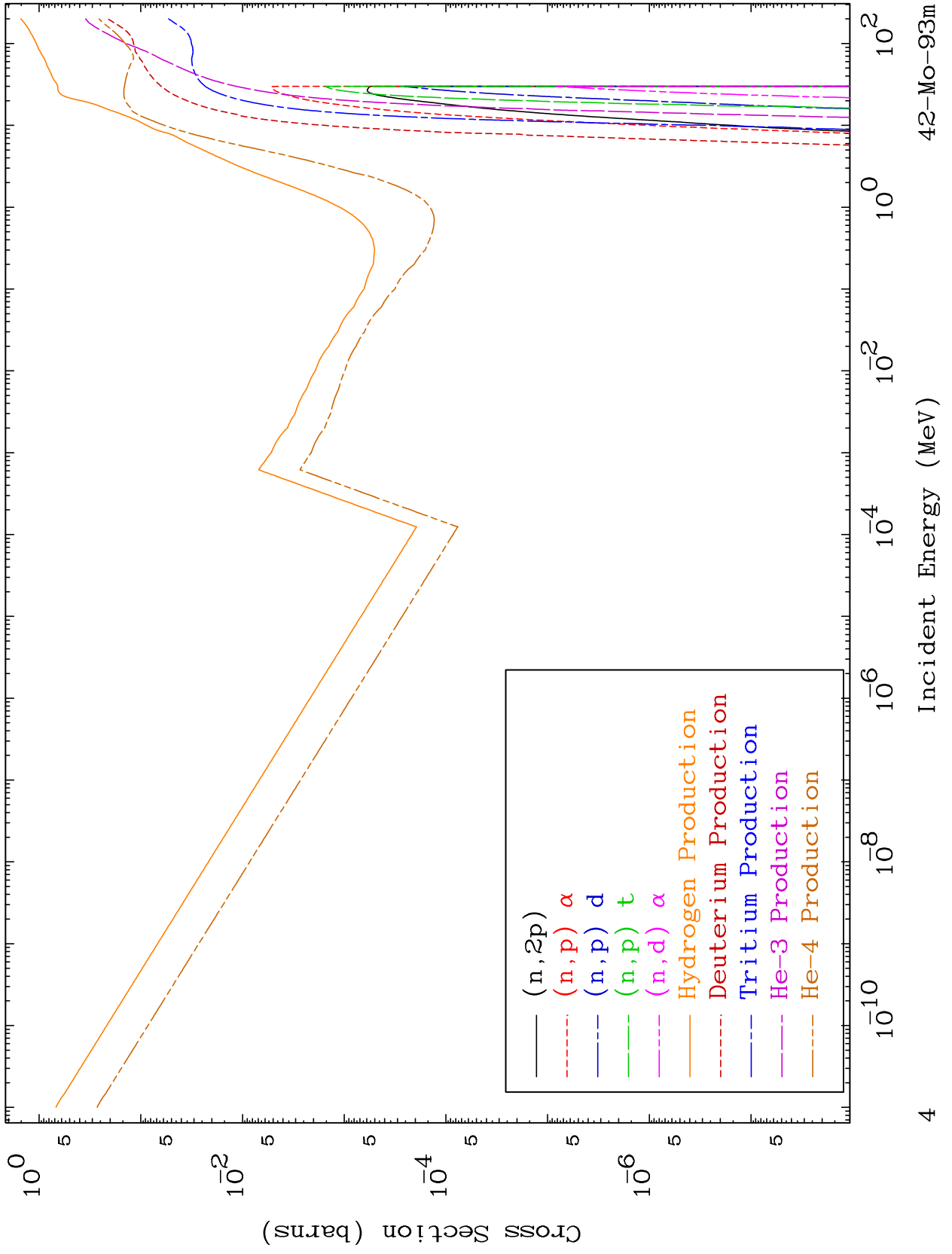
42-Mo-93m



MAT 4229

Neutron Absorption  
293 Kelvin Cross Sections

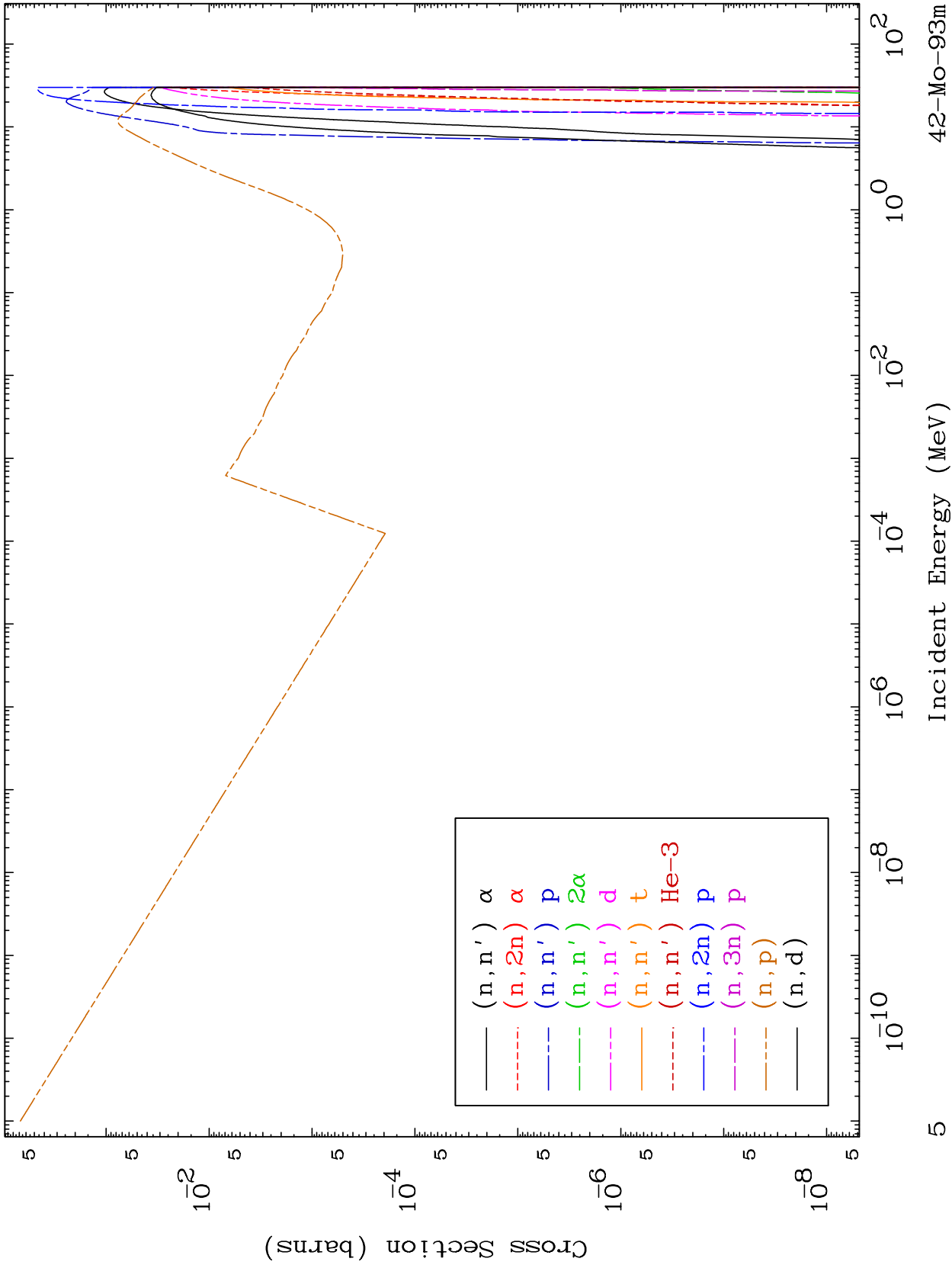
42-Mo-93m



MAT 4229

Charged Particle  
293 Kelvin Cross Sections

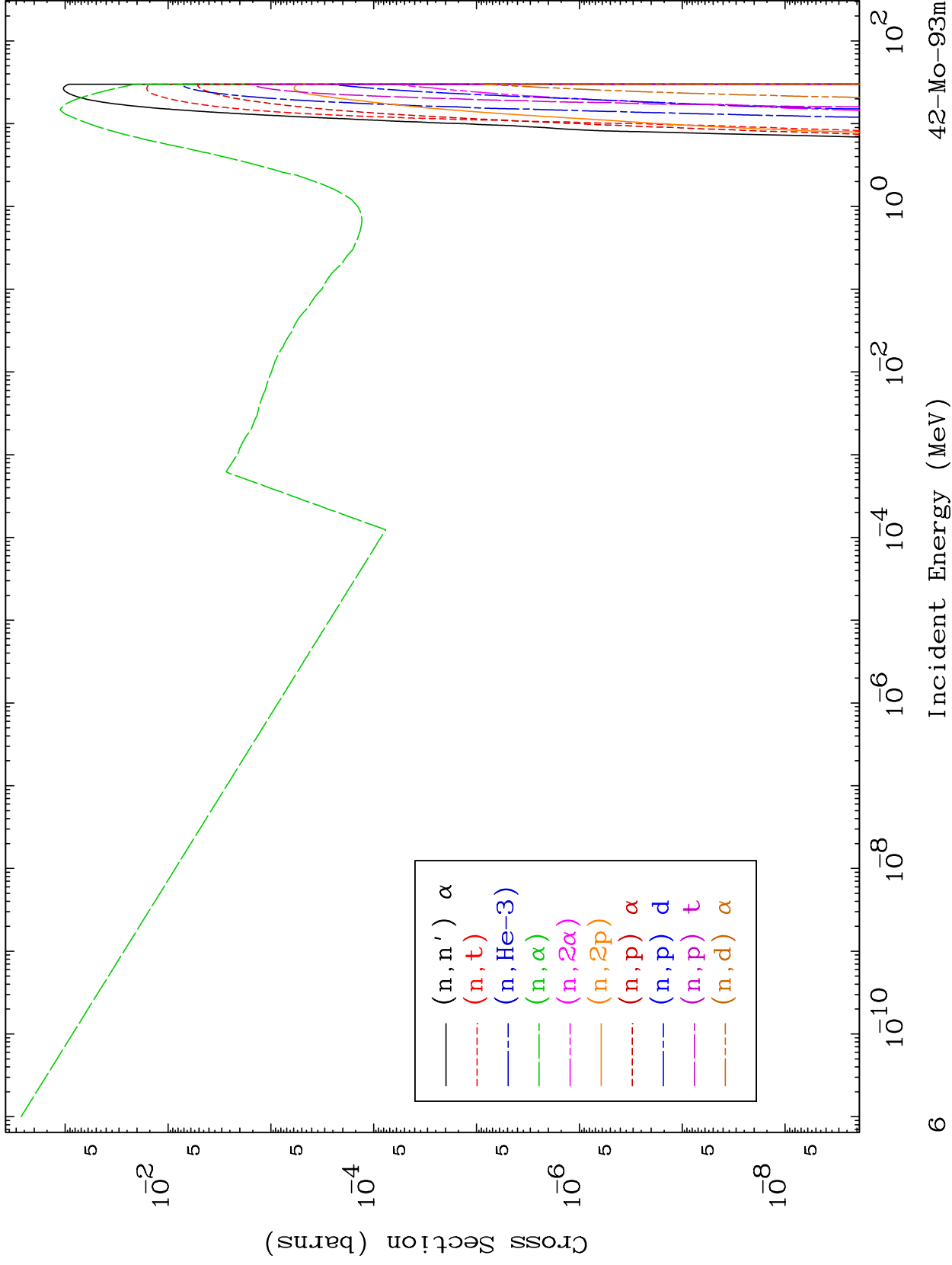
42-Mo-93m



MAT 4229

Charged Particle  
293 Kelvin Cross Sections

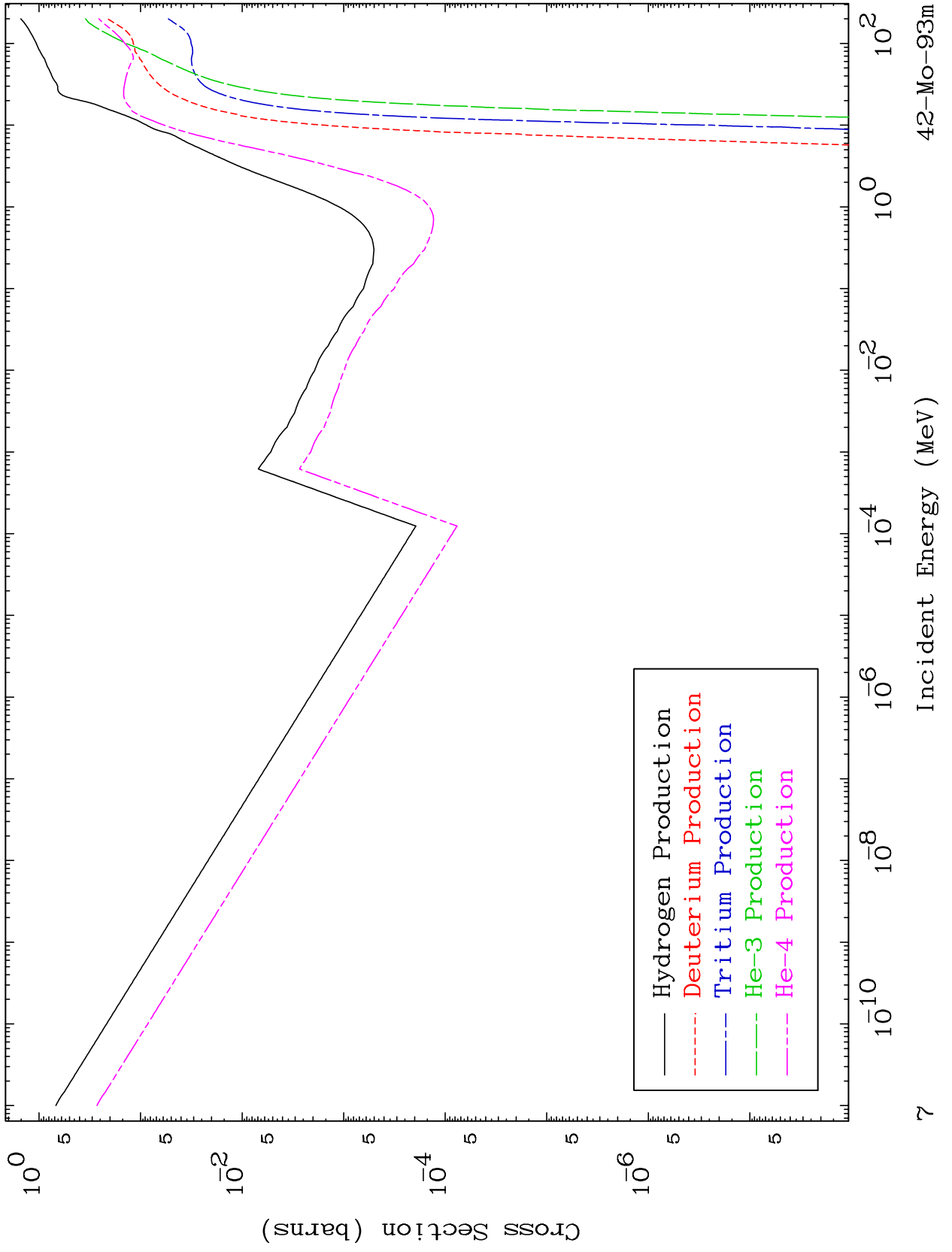
42-Mo-93m



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Particle Production  
293 Kelvin Cross Sections

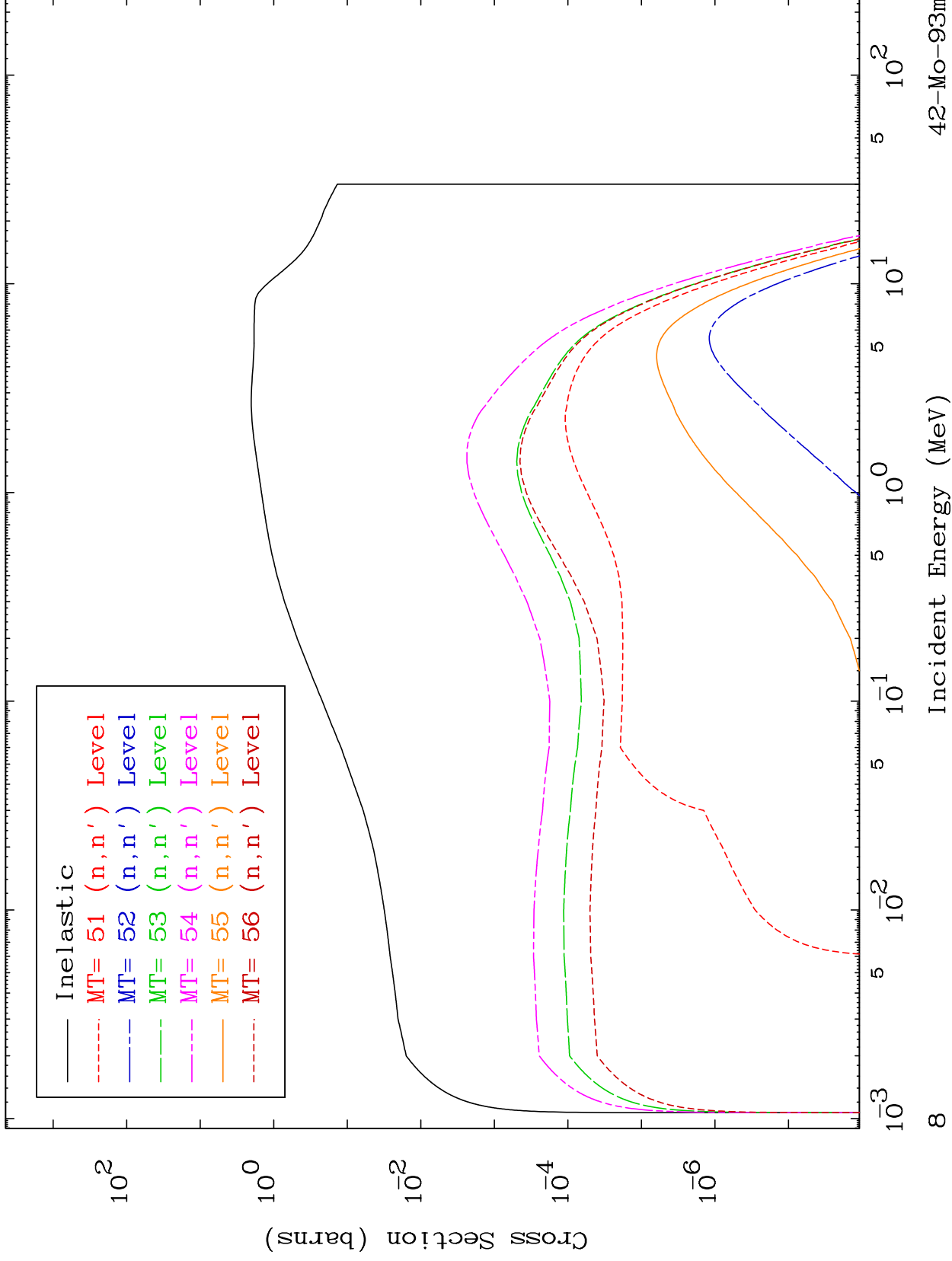
42-Mo-93m



MAT 4229

(n,n') Levels  
293 Kelvin Cross Sections

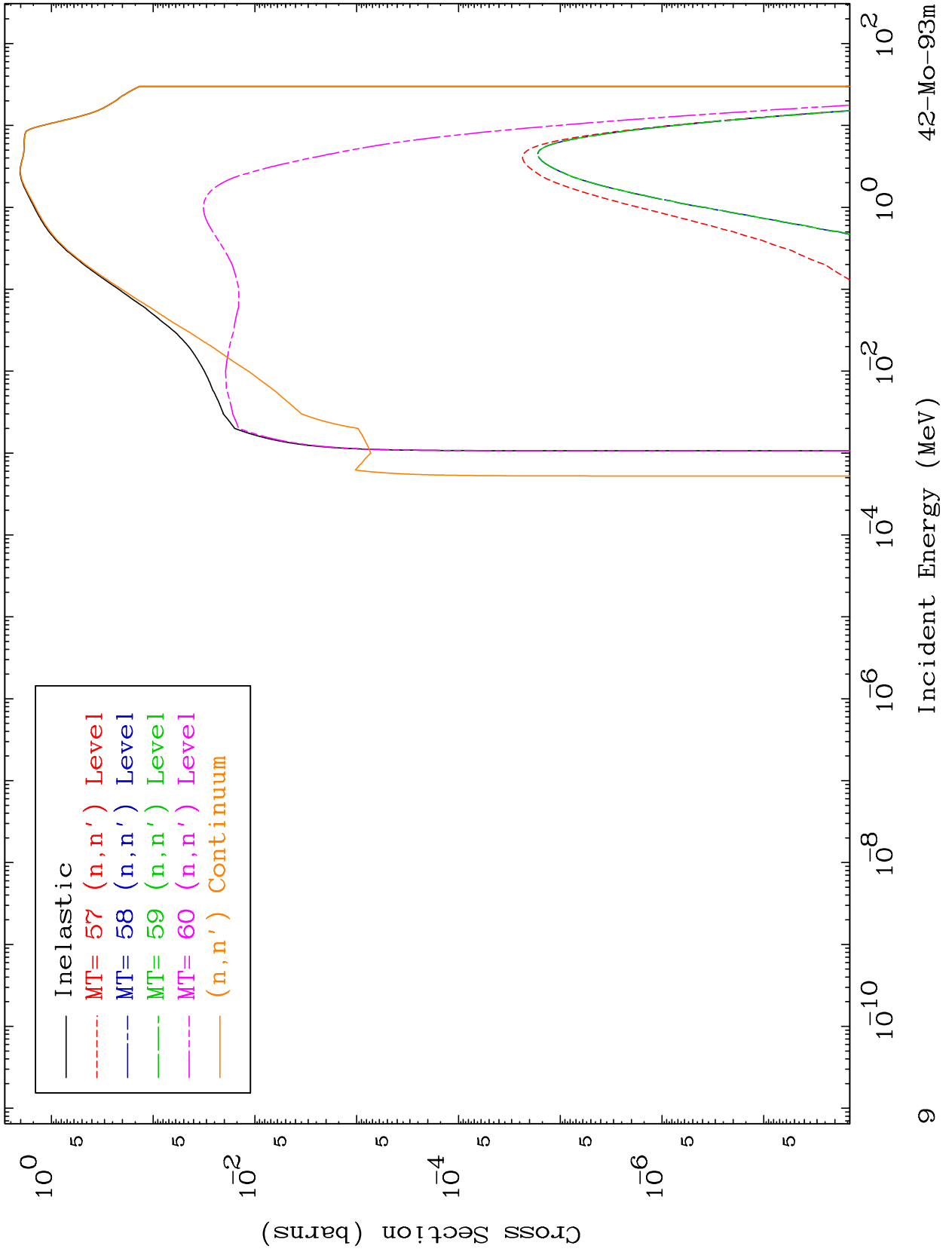
42-Mo-93m



MAT 4229

(n,n') Levels  
293 Kelvin Cross Sections

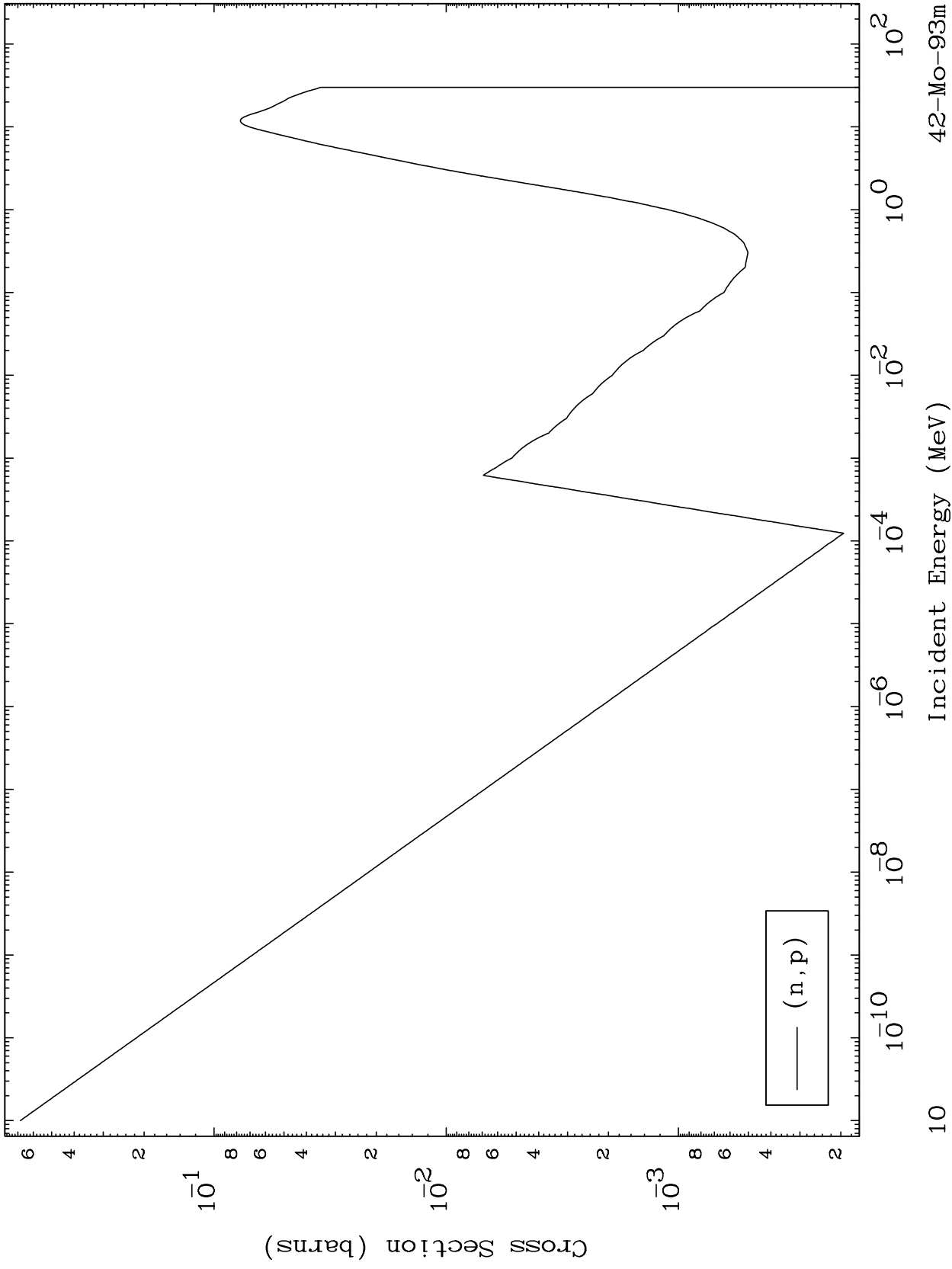
42-Mo-93m



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(n,p) Levels  
293 Kelvin Cross Sections

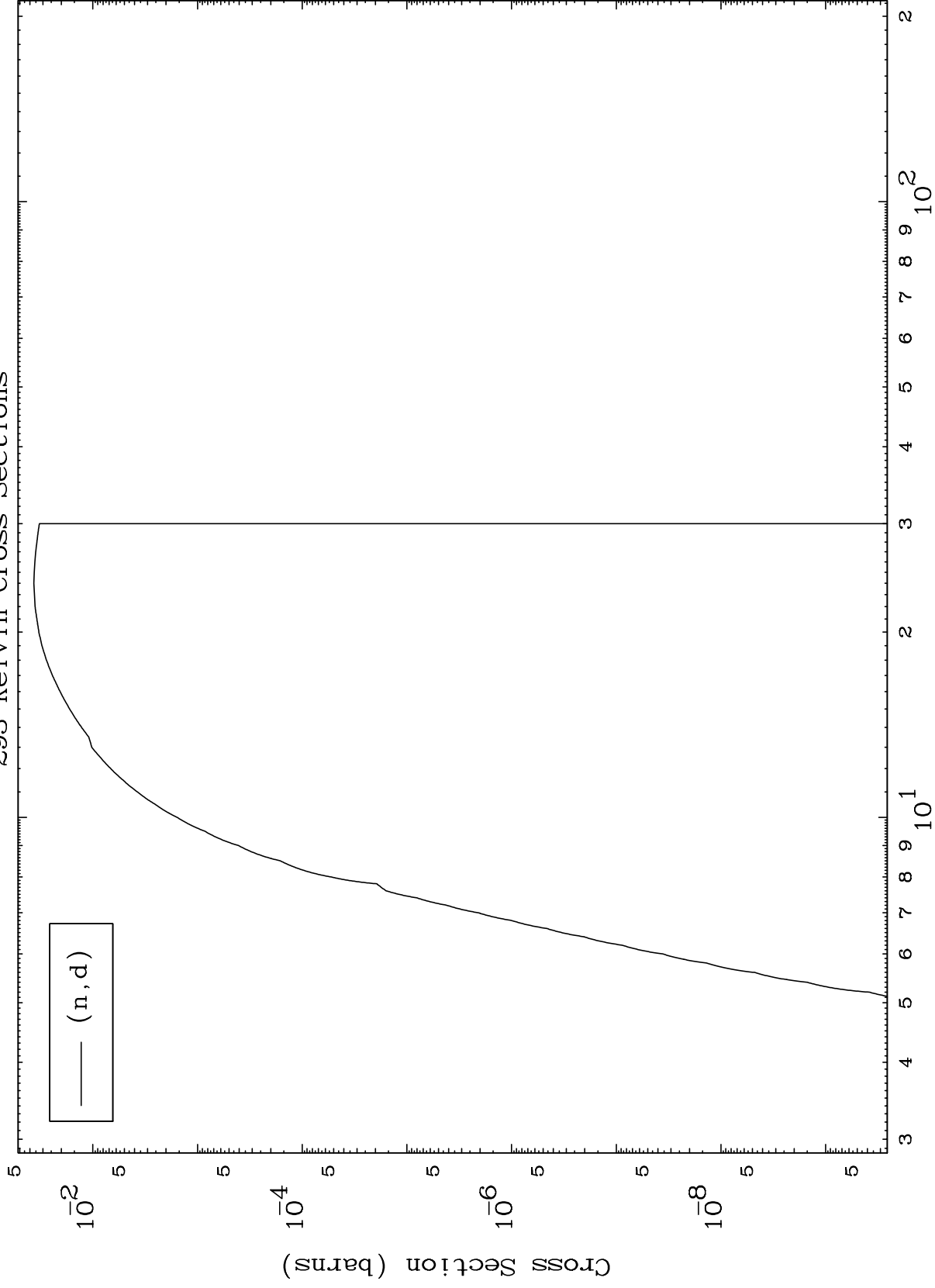
42-Mo-93m



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(n,d) Levels  
293 Kelvin Cross Sections

42-Mo-93m



Incident Energy (MeV)

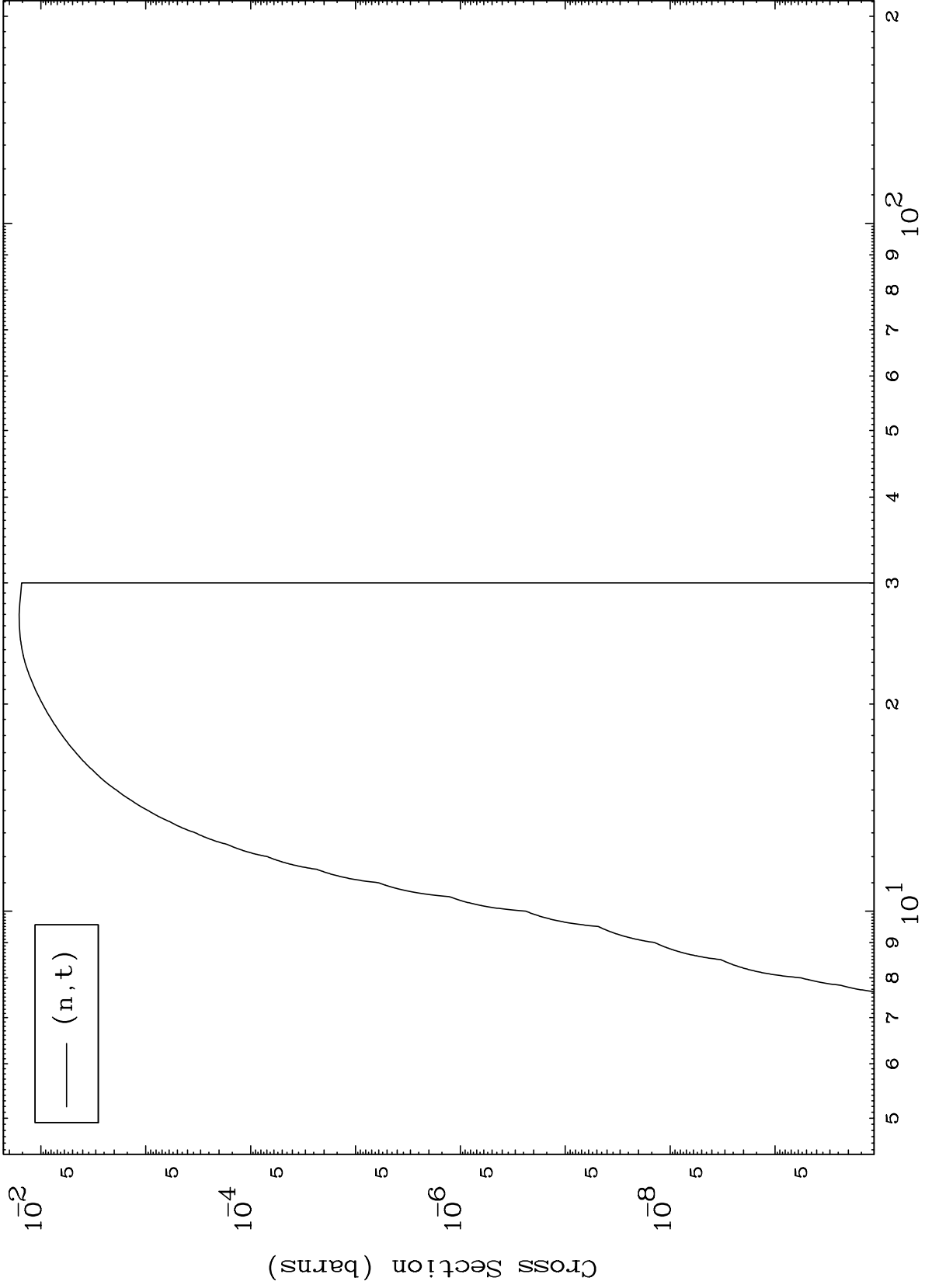
42-Mo-93m

11

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(n,t) Levels  
293 Kelvin Cross Sections

42-Mo-93m



12

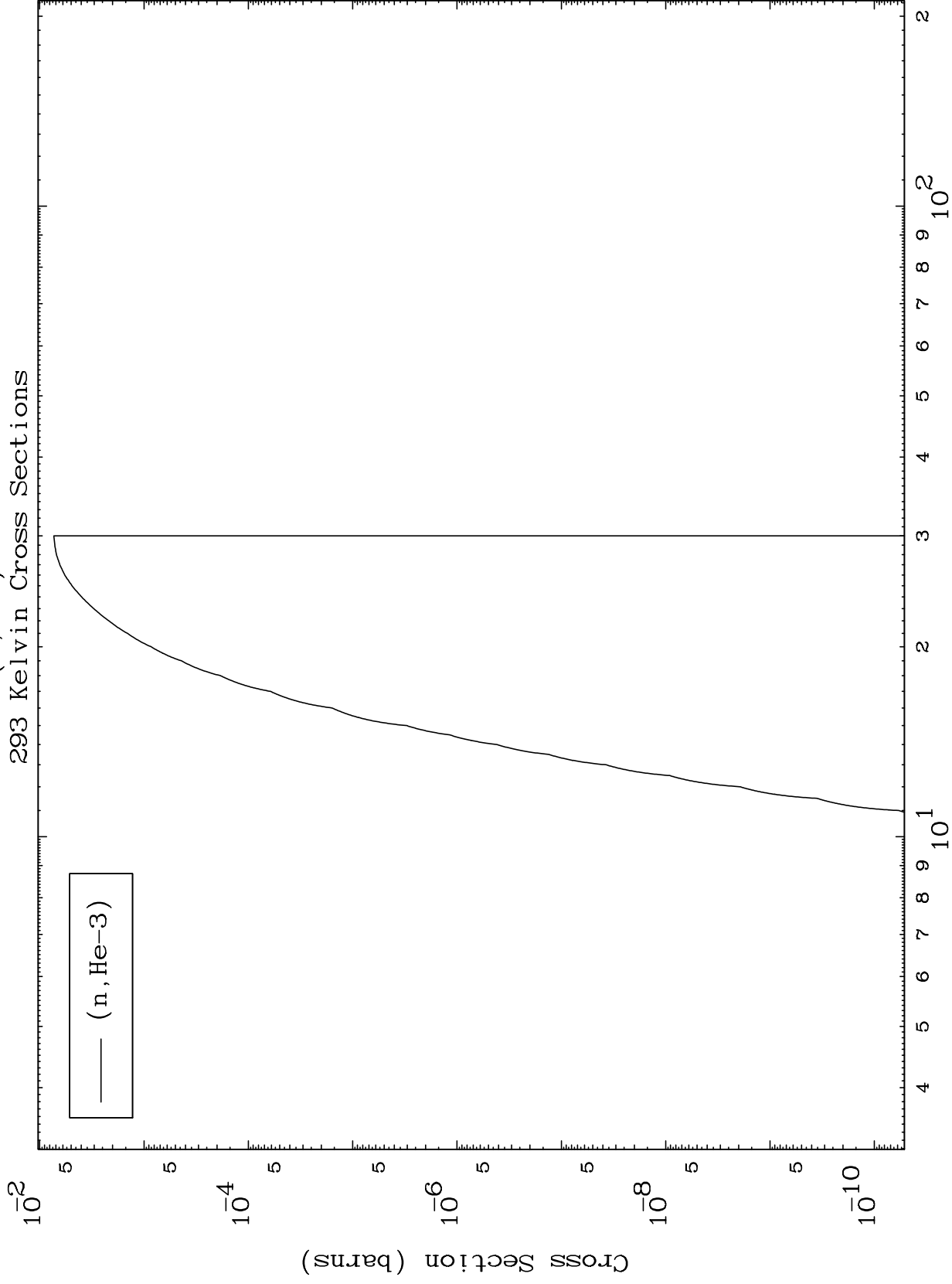
Incident Energy (MeV)

42-Mo-93m

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(n,He3) Levels  
293 Kelvin Cross Sections

42-Mo-93m



13

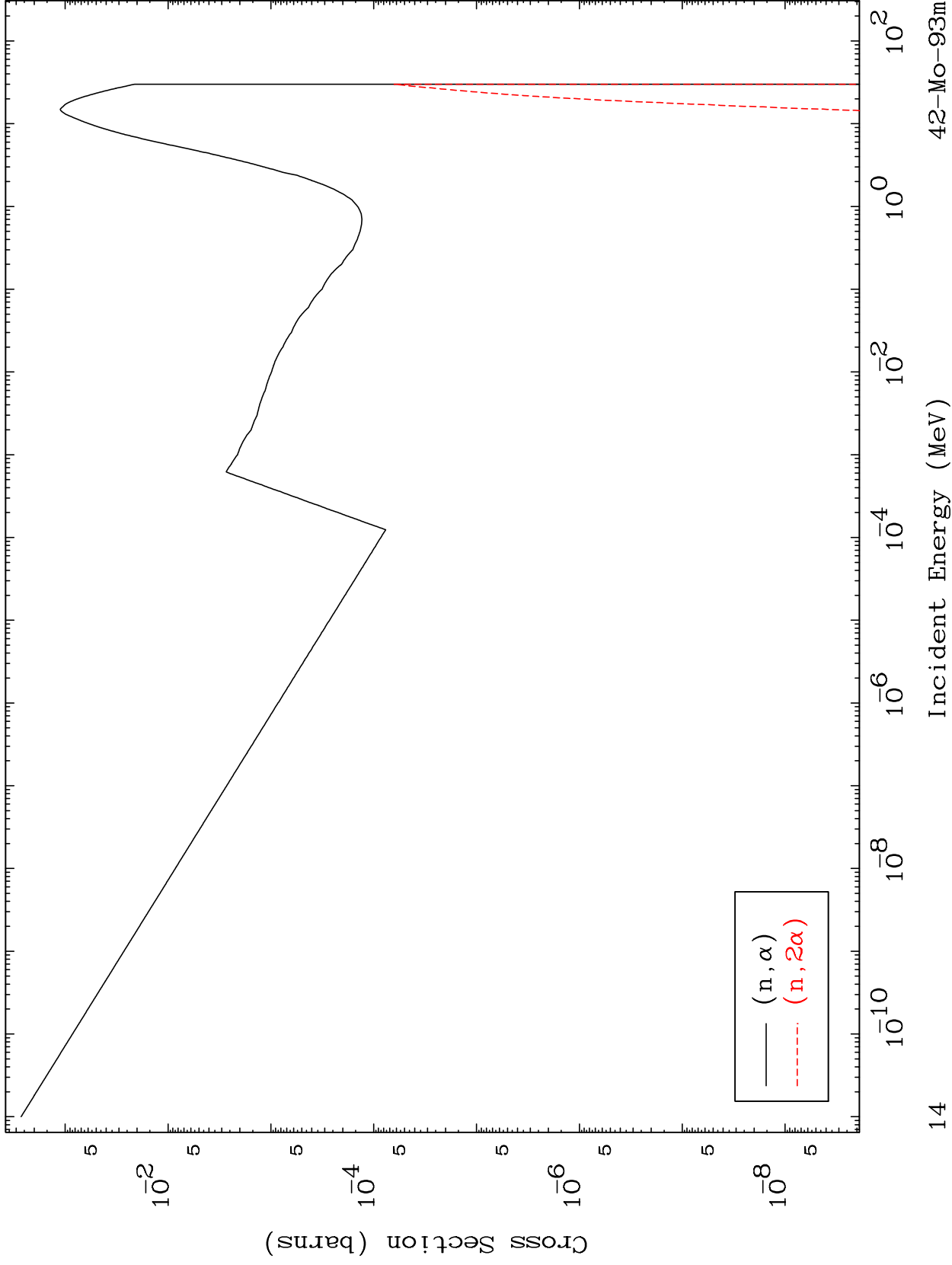
Incident Energy (MeV)

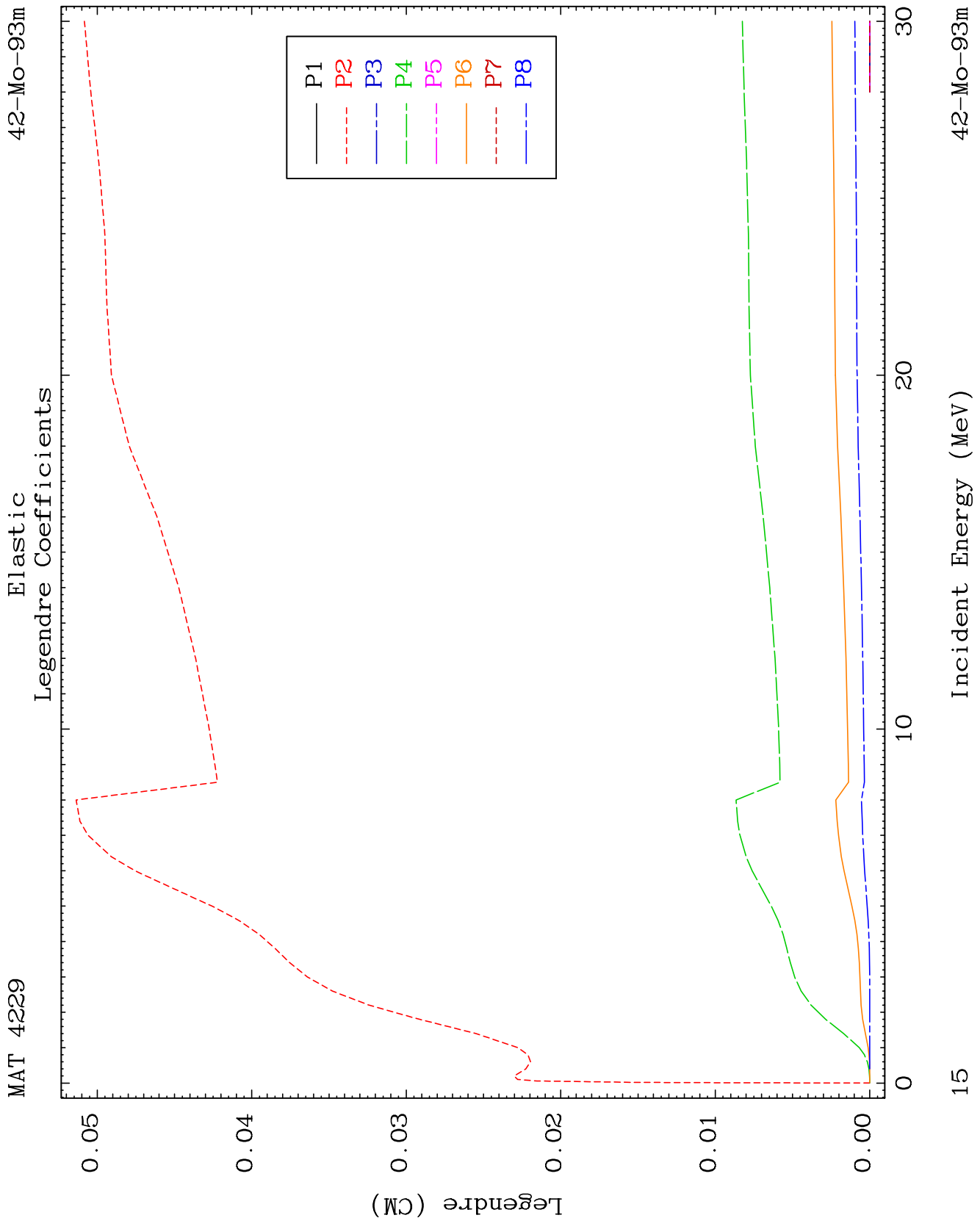
42-Mo-93m

MAT 4229

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

42-Mo-93m

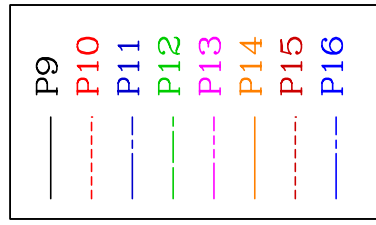




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Elastic Legendre Coefficients

42-Mo-93m



$\times 10^{-4}$

Legendre (CM)



16

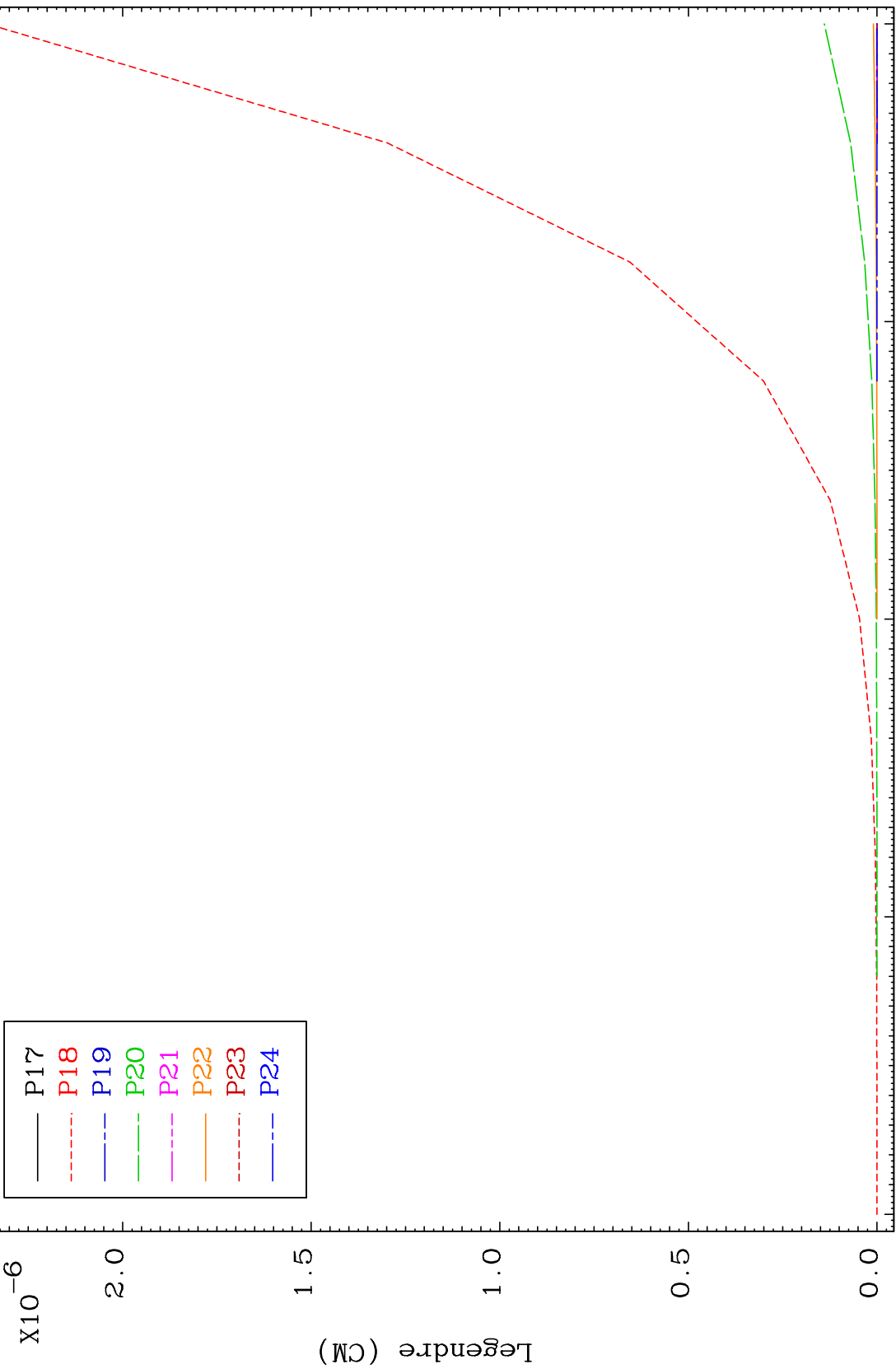
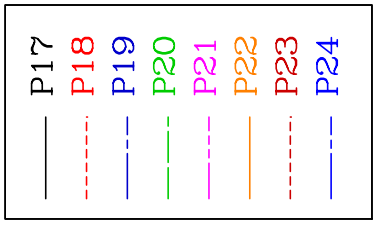
Incident Energy (MeV)

42-Mo-93m

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### Elastic Legendre Coefficients

42-Mo-93m



17

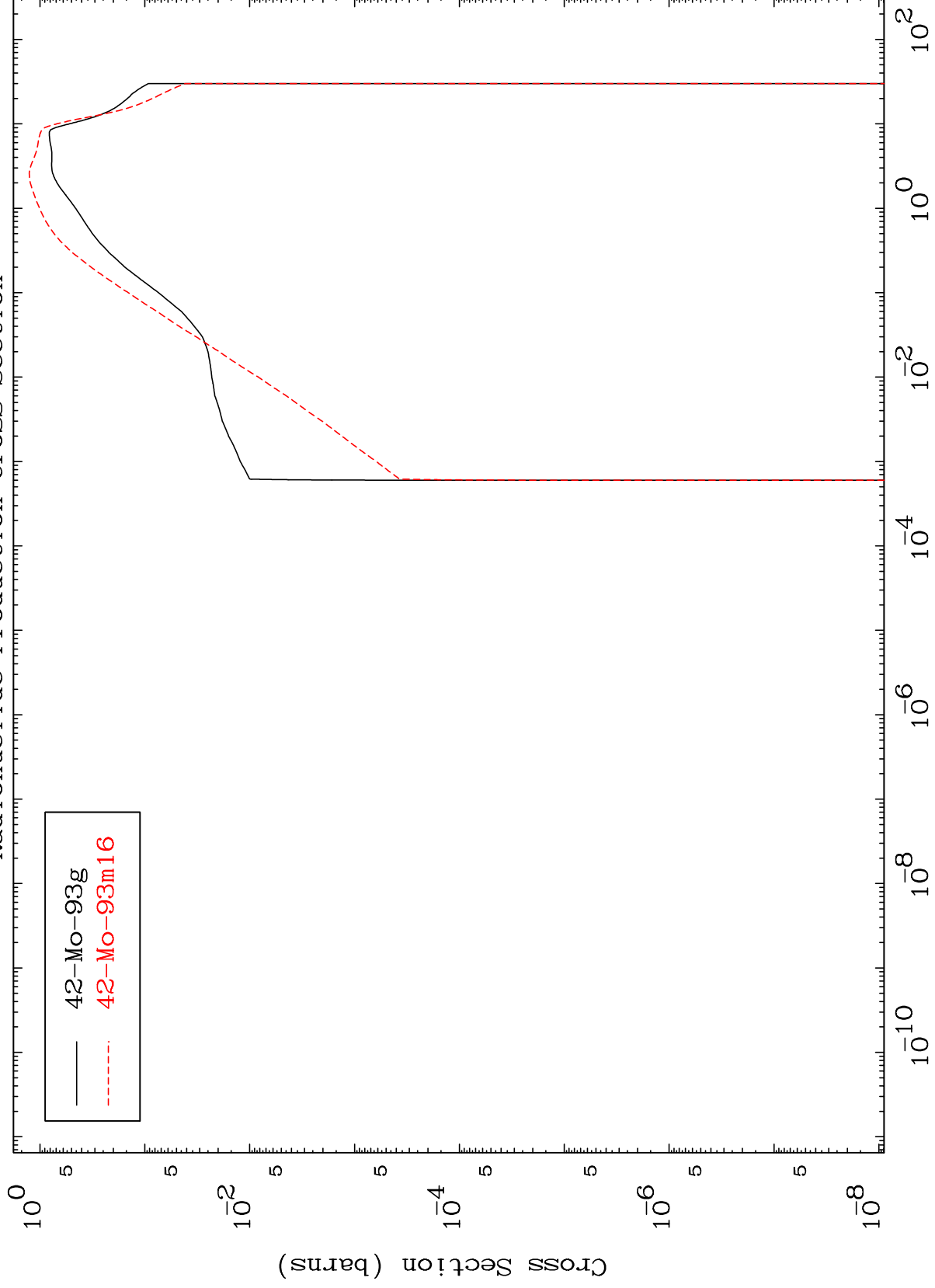
Incident Energy (MeV)

42-Mo-93m

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Inelastic  
Radionuclide Production Cross Section

42-Mo-93m



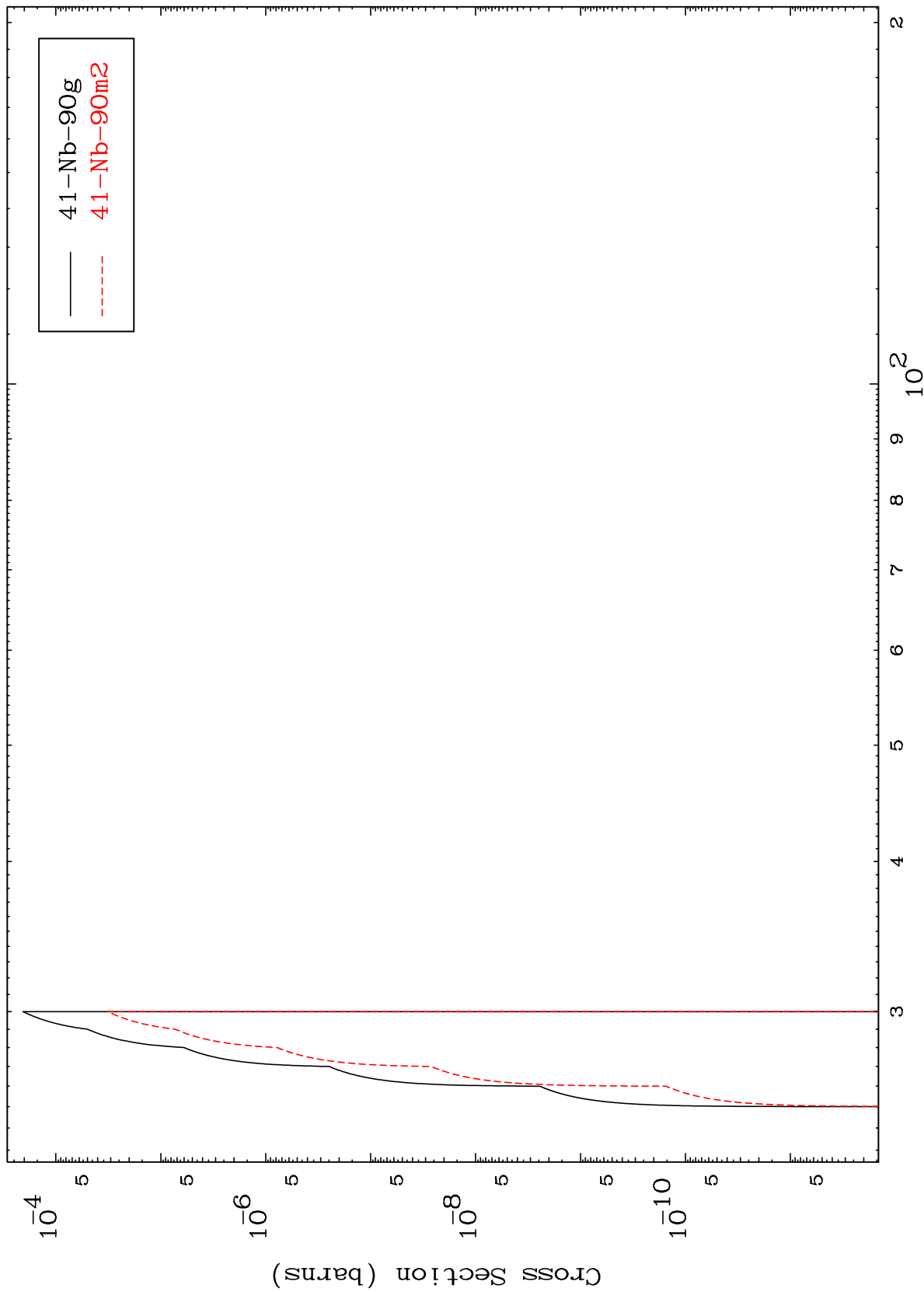
42-Mo-93g  
42-Mo-93m16

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(n,2n) d

42-Mo-93m

Radionuclide Production Cross Section



19

Incident Energy (MeV)

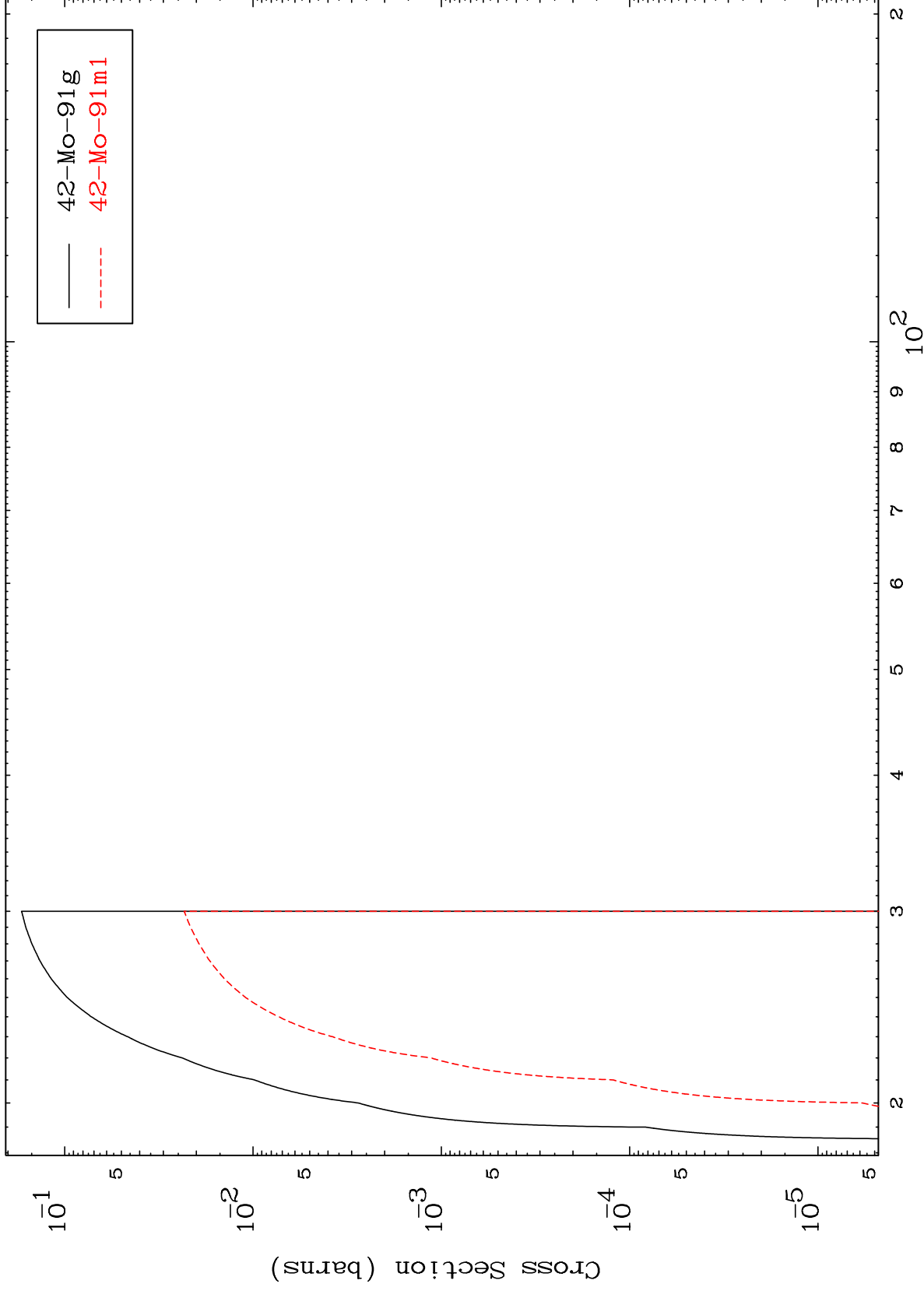
42-Mo-93m

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(n,3n)

42-Mo-93m

Radionuclide Production Cross Section



42-Mo-91g  
42-Mo-91m1

20

Incident Energy (MeV)

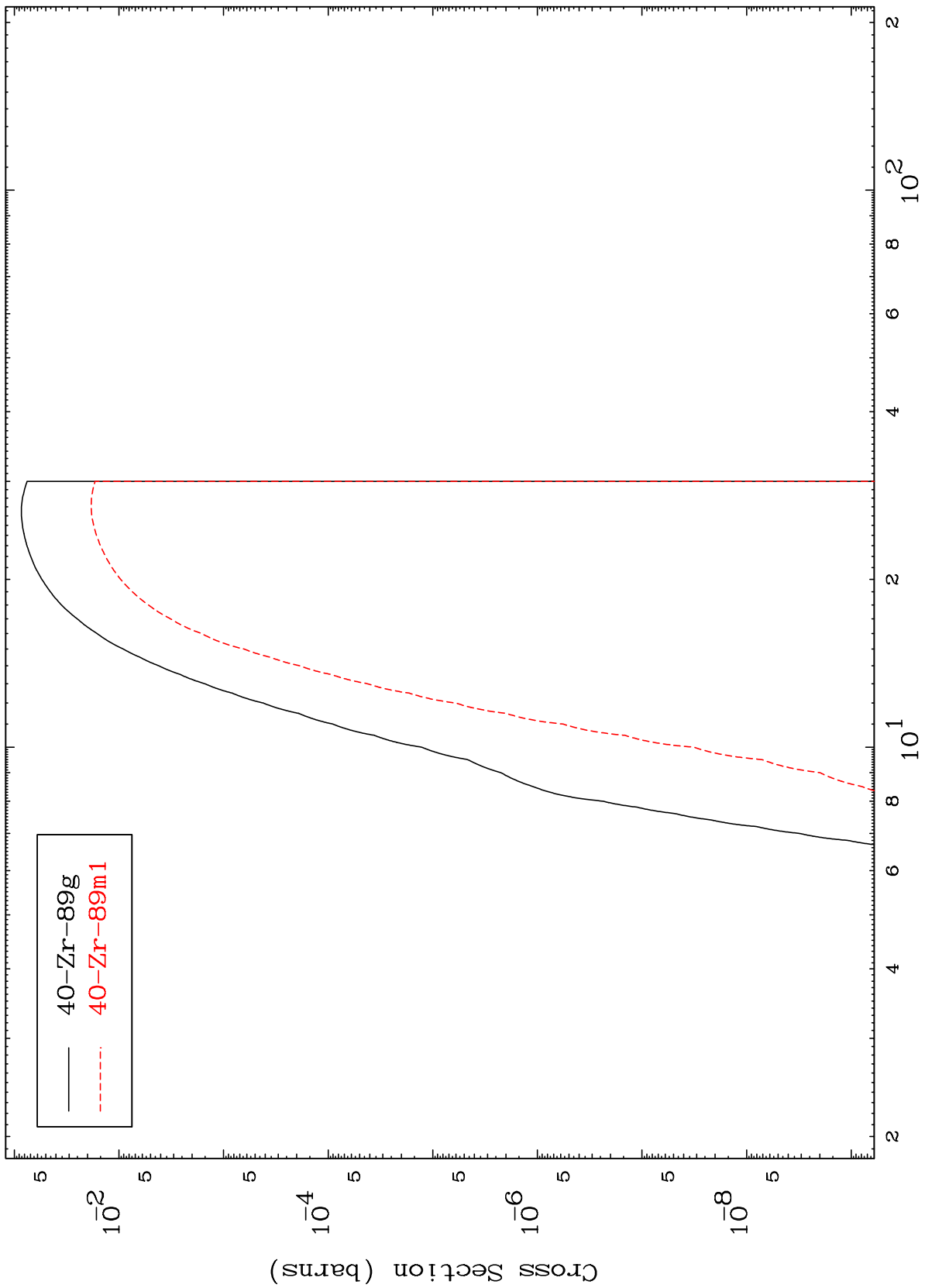
42-Mo-93m

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$(n, n') \alpha$

42-Mo-93m

Radionuclide Production Cross Section



21

Incident Energy (MeV)

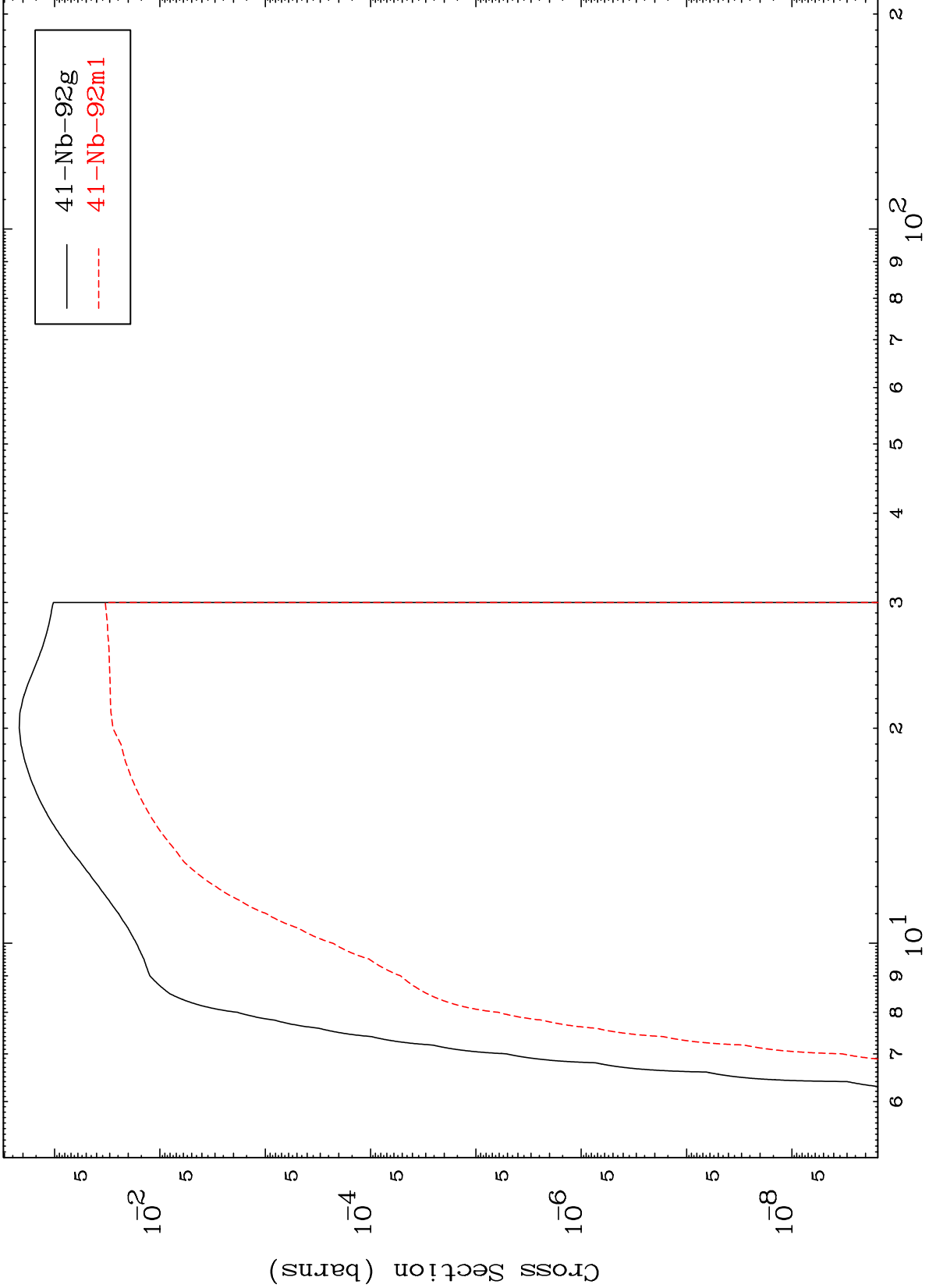
42-Mo-93m

MAT 4229

(n,n') p

42-Mo-93m

Radionuclide Production Cross Section



22

Incident Energy (MeV)

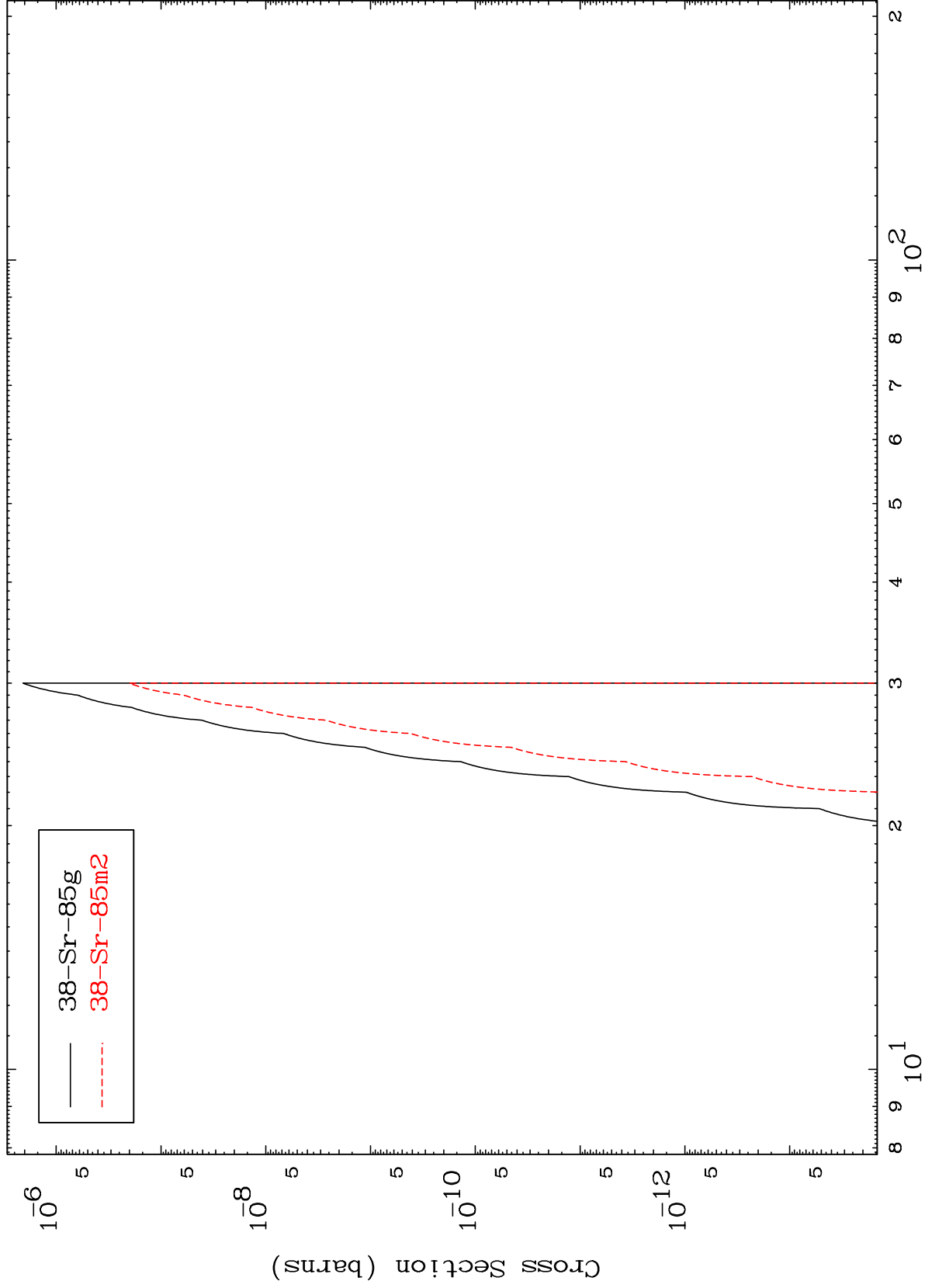
42-Mo-93m

MAT 4229

(n,n') 2α

42-Mo-93m

Radionuclide Production Cross Section



23

Incident Energy (MeV)

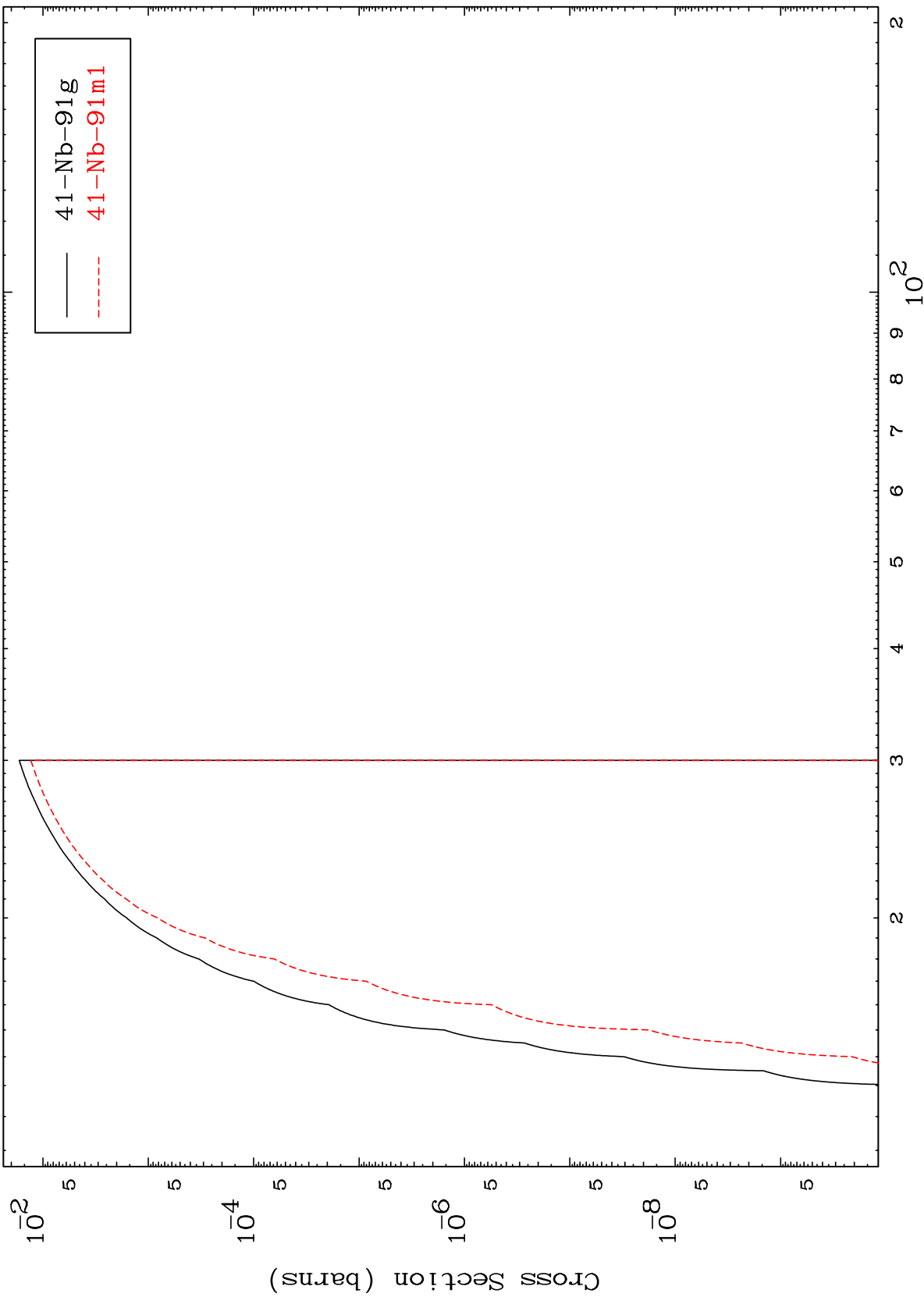
42-Mo-93m

MAT 4229

(n,n') d

42-Mo-93m

Radionuclide Production Cross Section

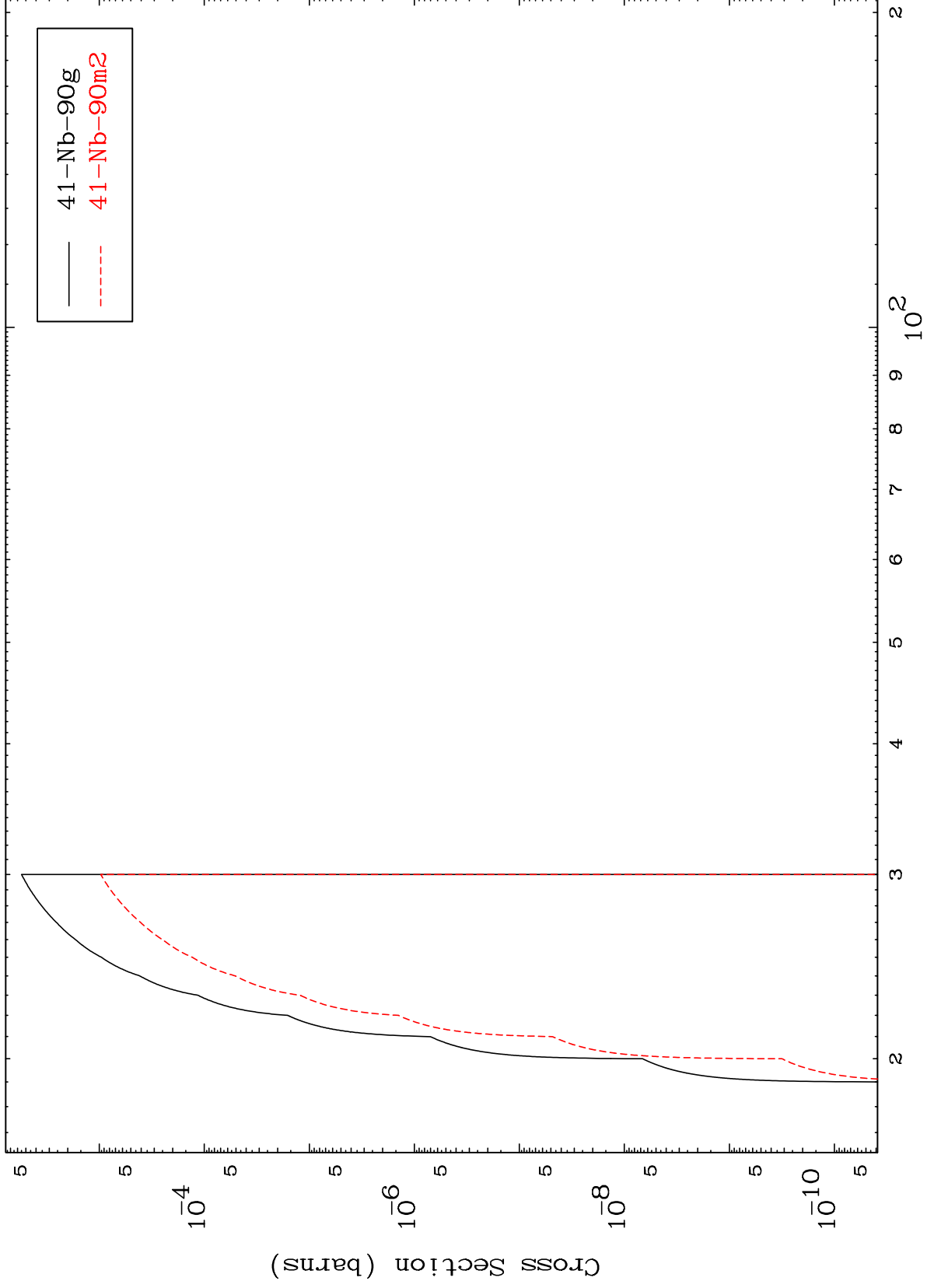


24

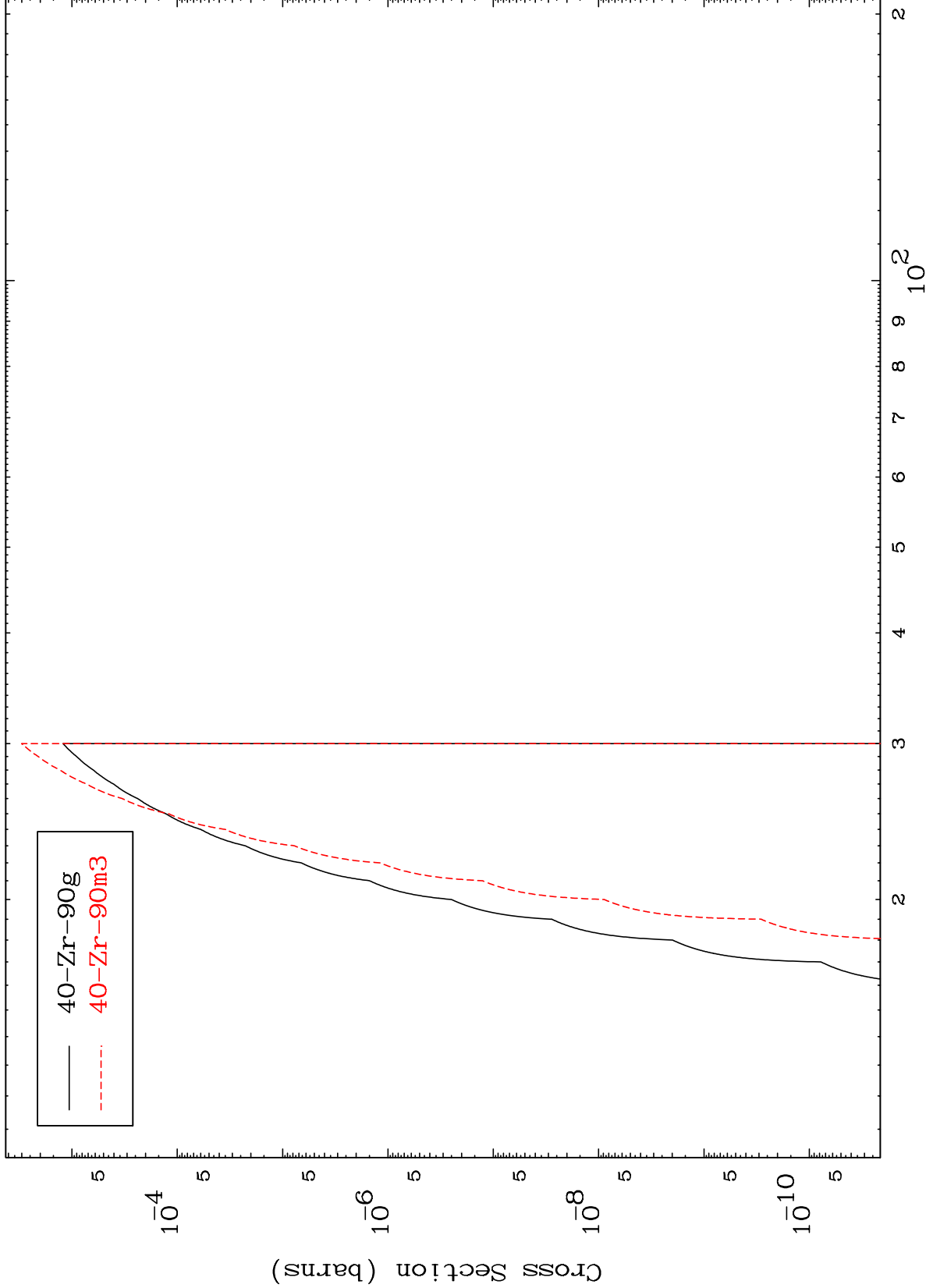
Incident Energy (MeV)

42-Mo-93m

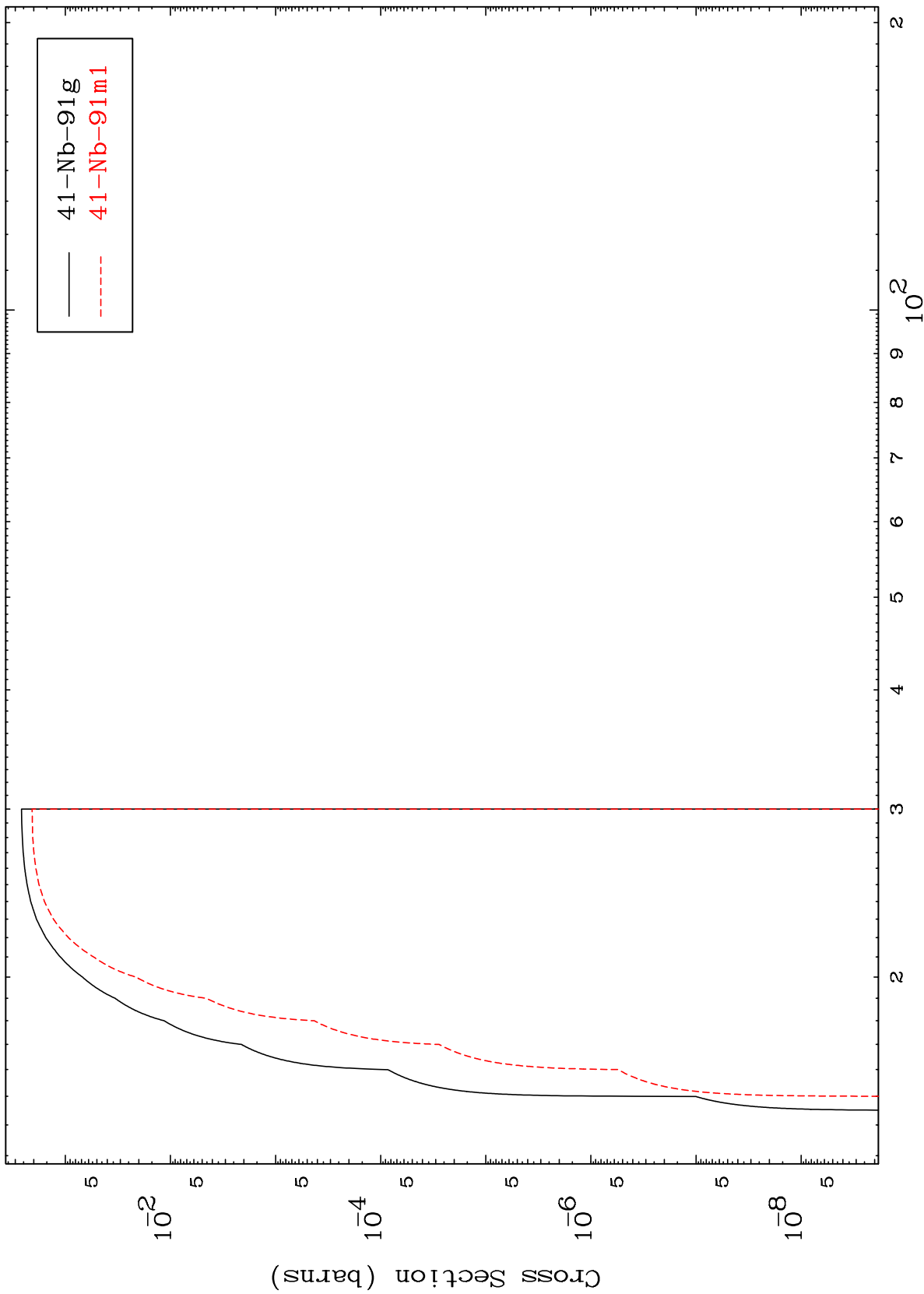
Radionuclide Production Cross Section



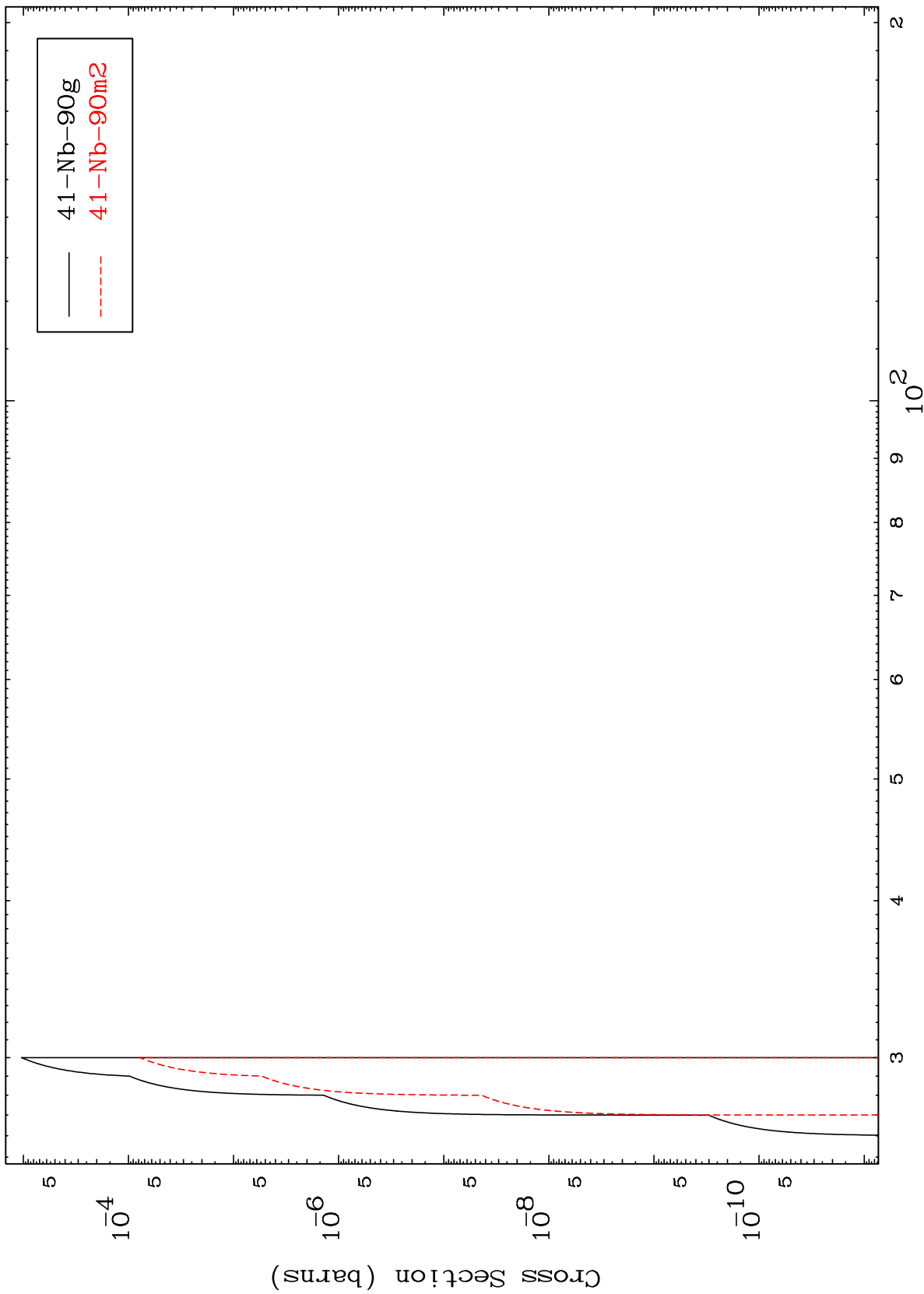
Radionuclide Production Cross Section



Radionuclide Production Cross Section



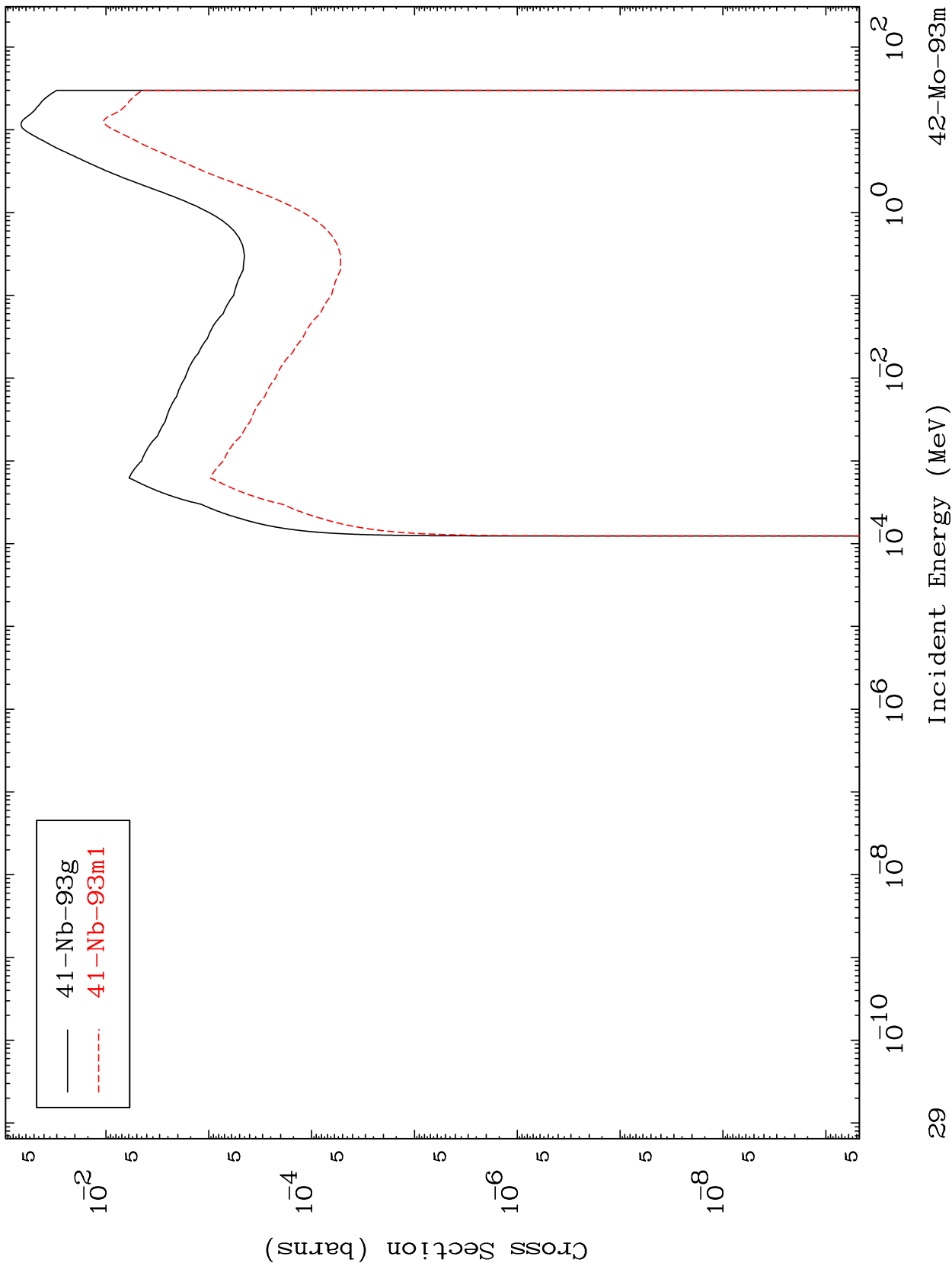
Radionuclide Production Cross Section



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42-Mo-93m

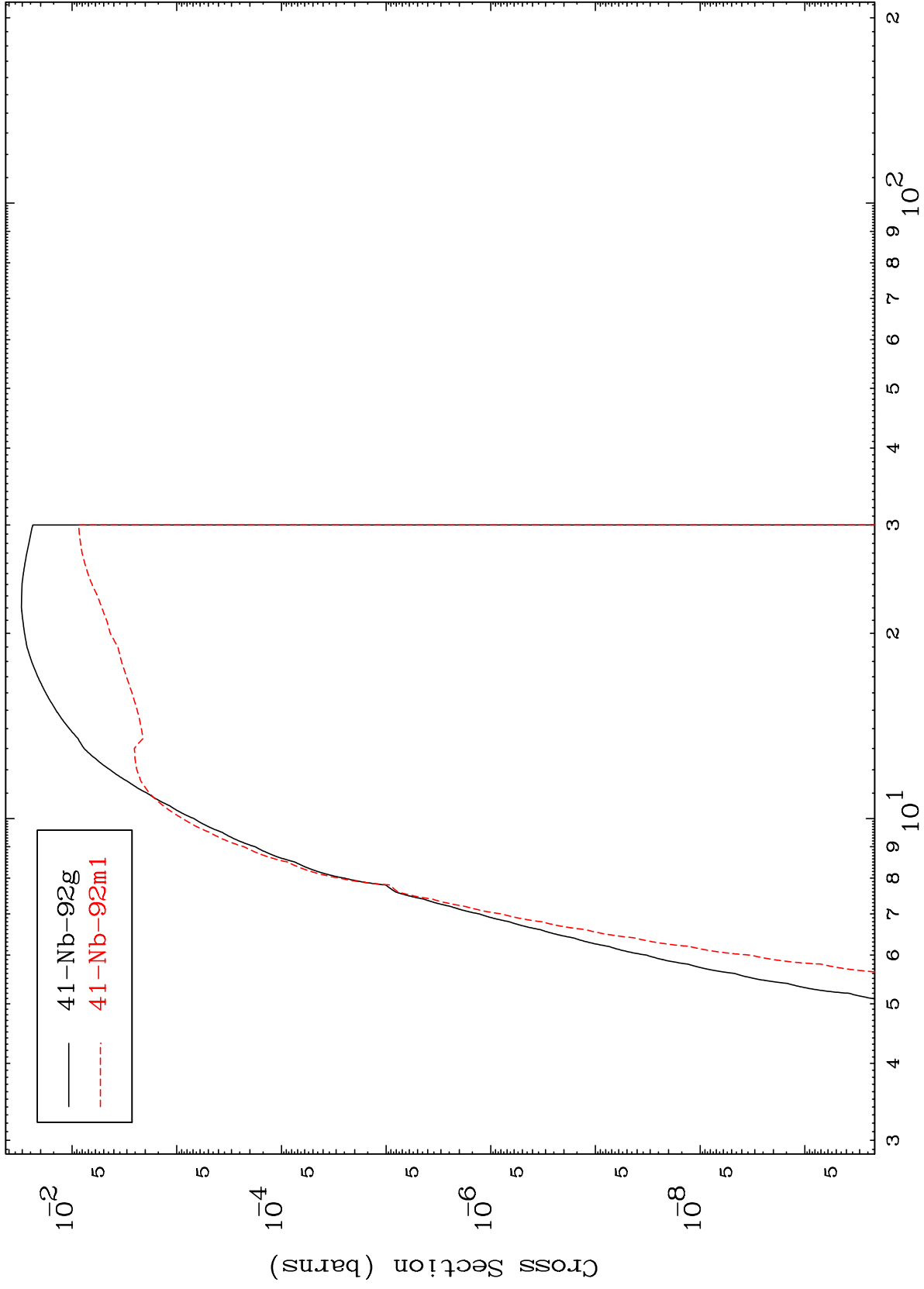
Radionuclide Production Cross Section



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42-Mo-93m

(n,d)  
Radionuclide Production Cross Section



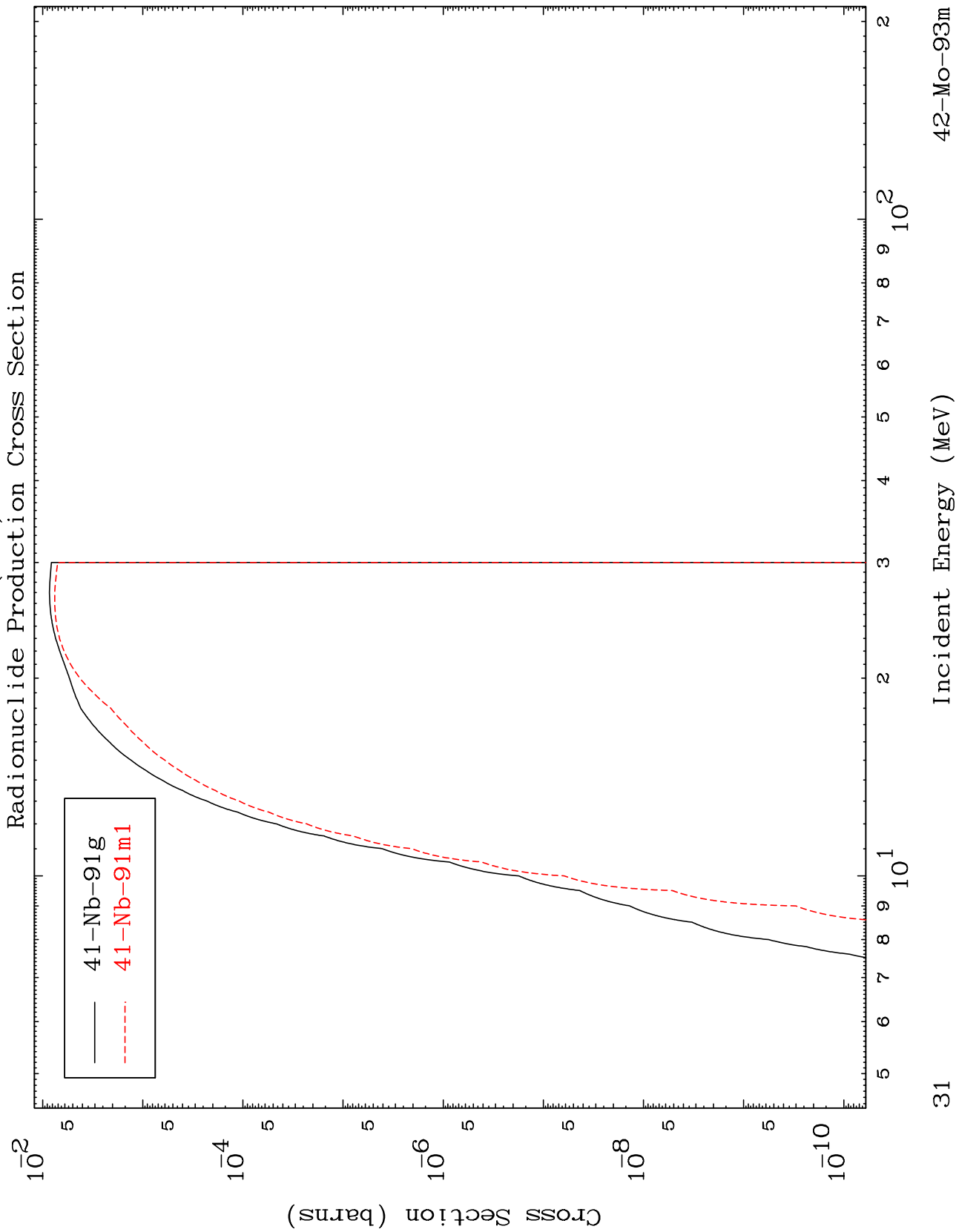
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Incident Energy (MeV)

42-Mo-93m

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42-Mo-93m



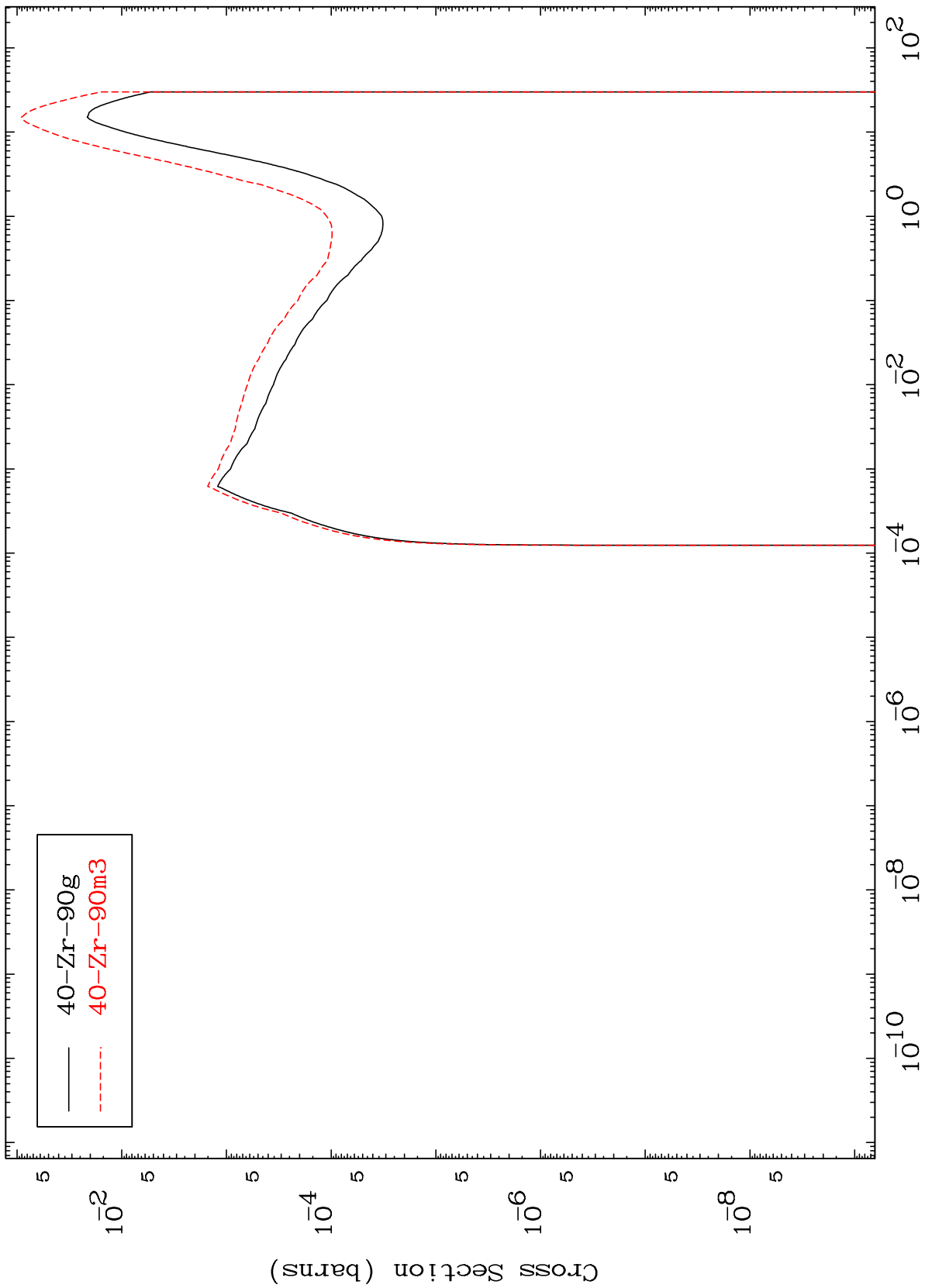
31

42-Mo-93m

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42-Mo-93m

Radionuclide Production Cross Section

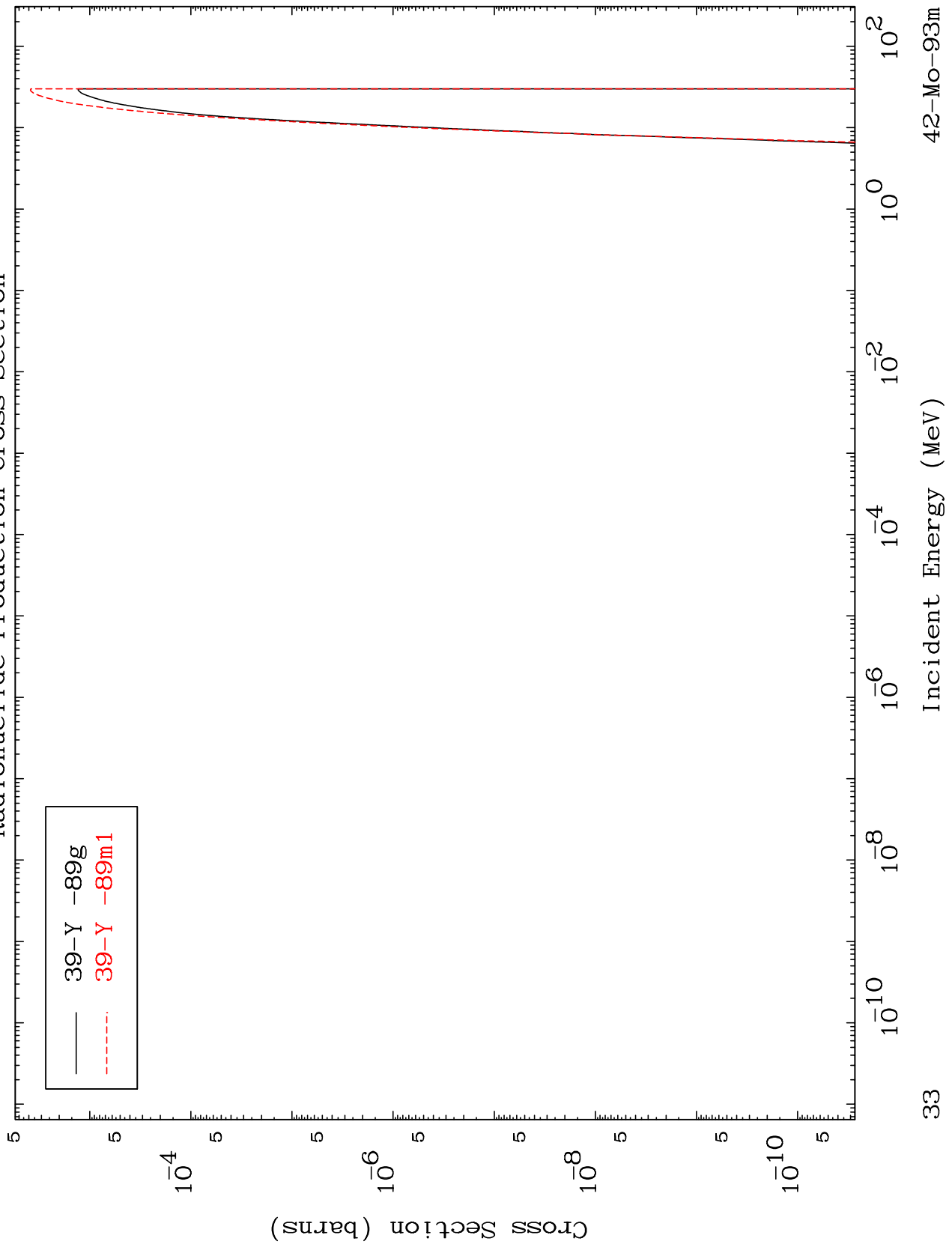


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(n,p)  $\alpha$

42-Mo-93m

Radionuclide Production Cross Section

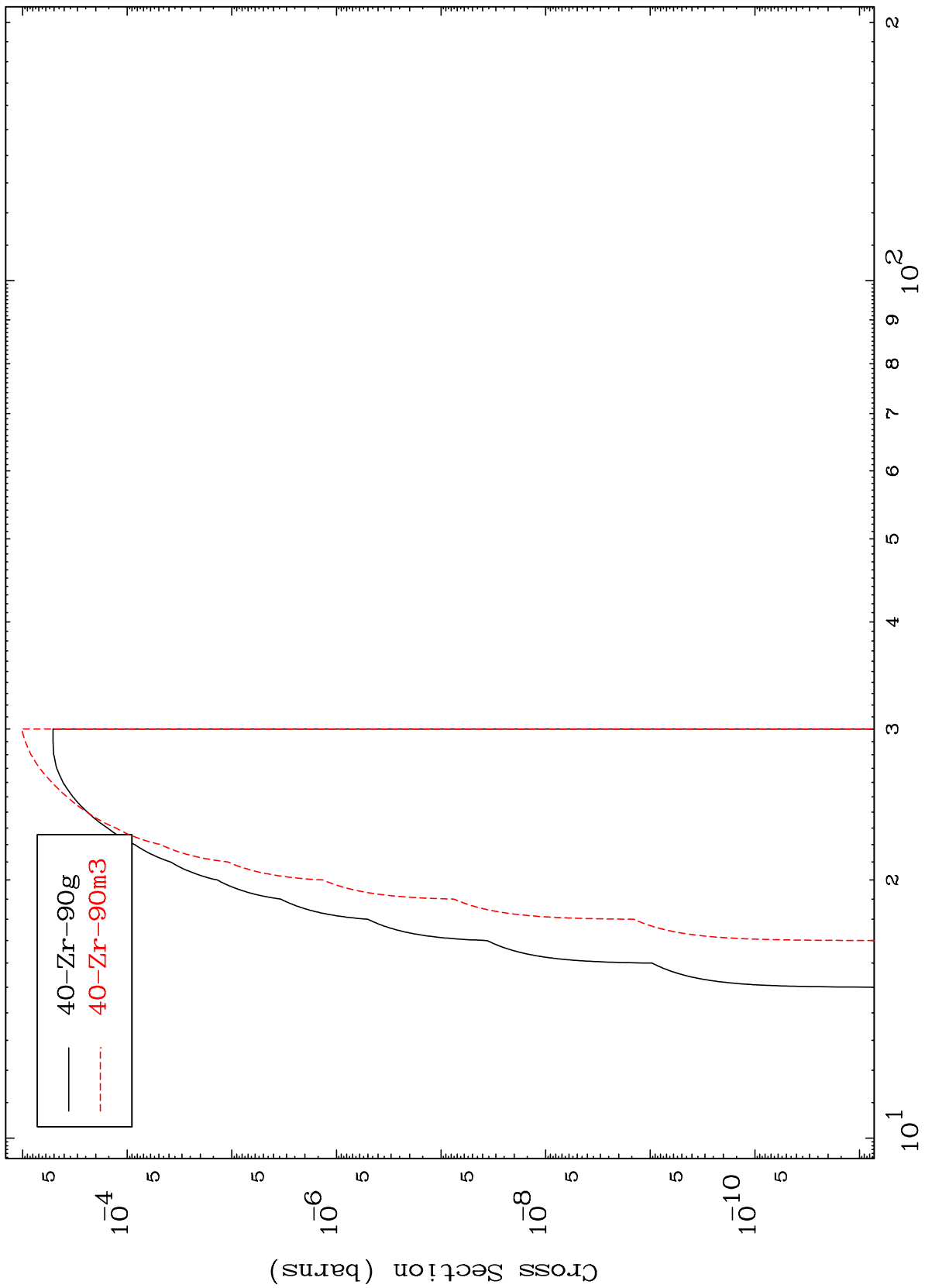


MAT 4229

(n,p) t

42-Mo-93m

Radionuclide Production Cross Section



34

Incident Energy (MeV)

42-Mo-93m