

Program EVALPLOT  
(Version 2021-1)

by

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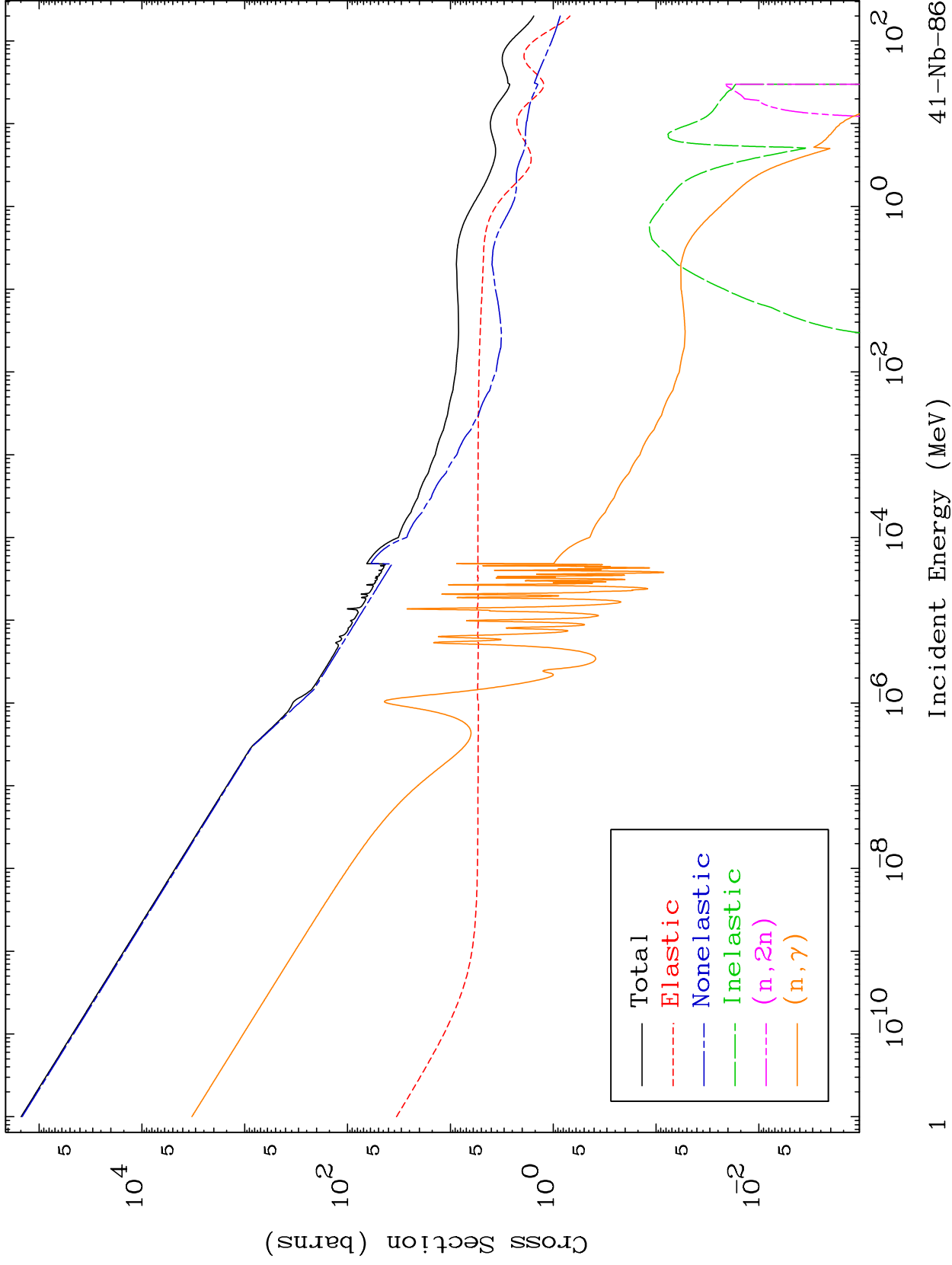
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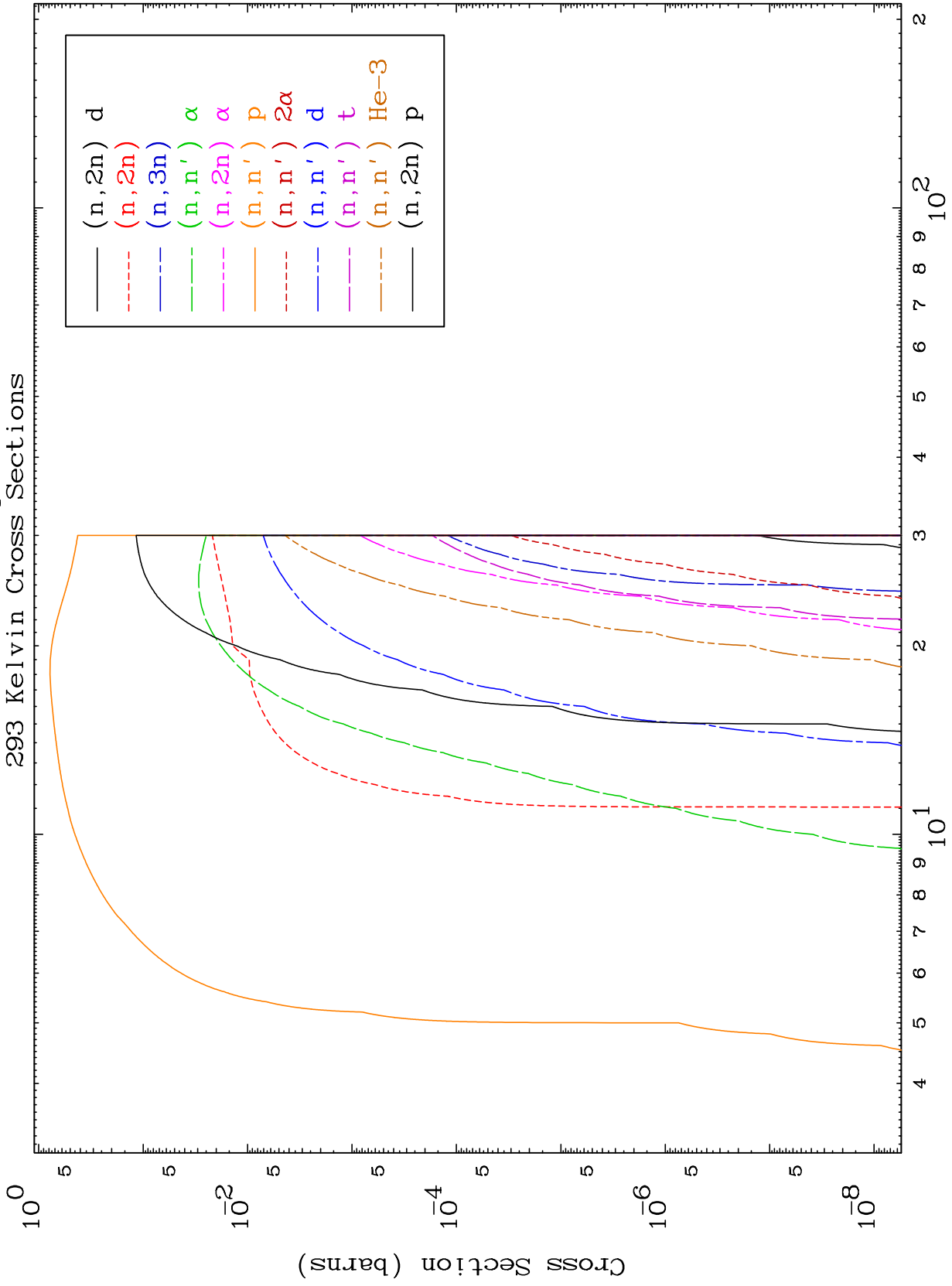
Press Mouse Button to Start

MAT 4104

Neutron Major  
293 Kelvin Cross Sections

41-Nb-86

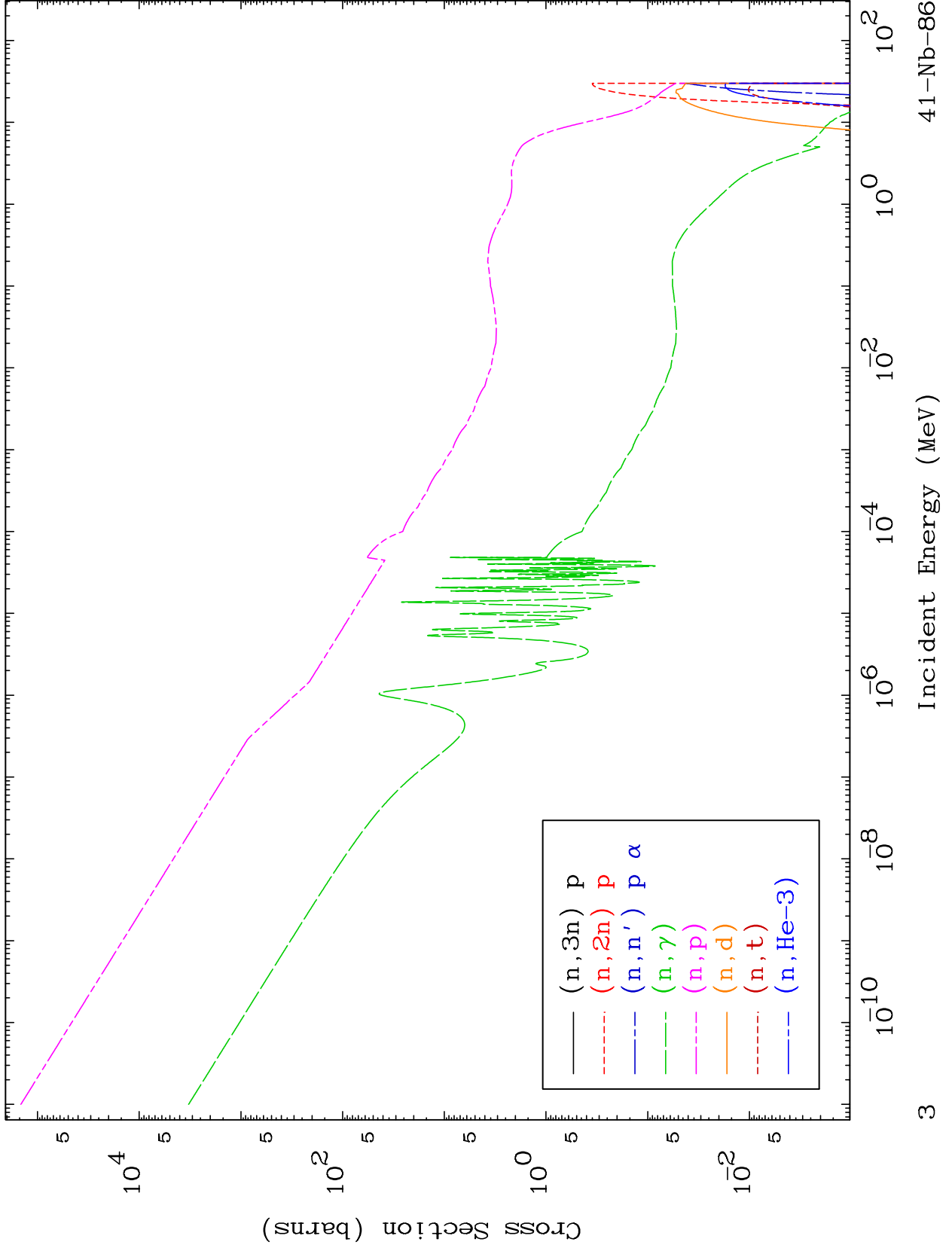


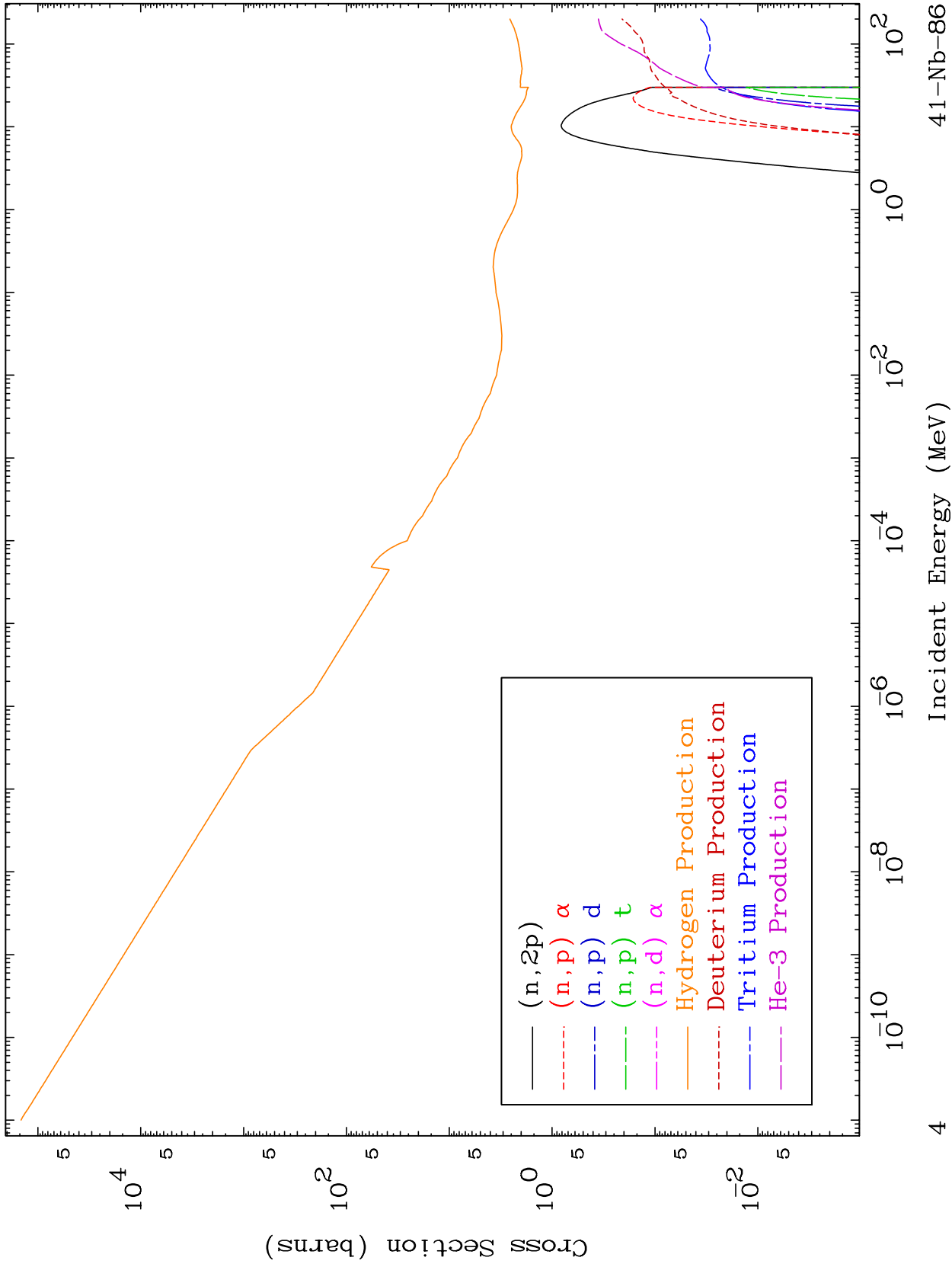


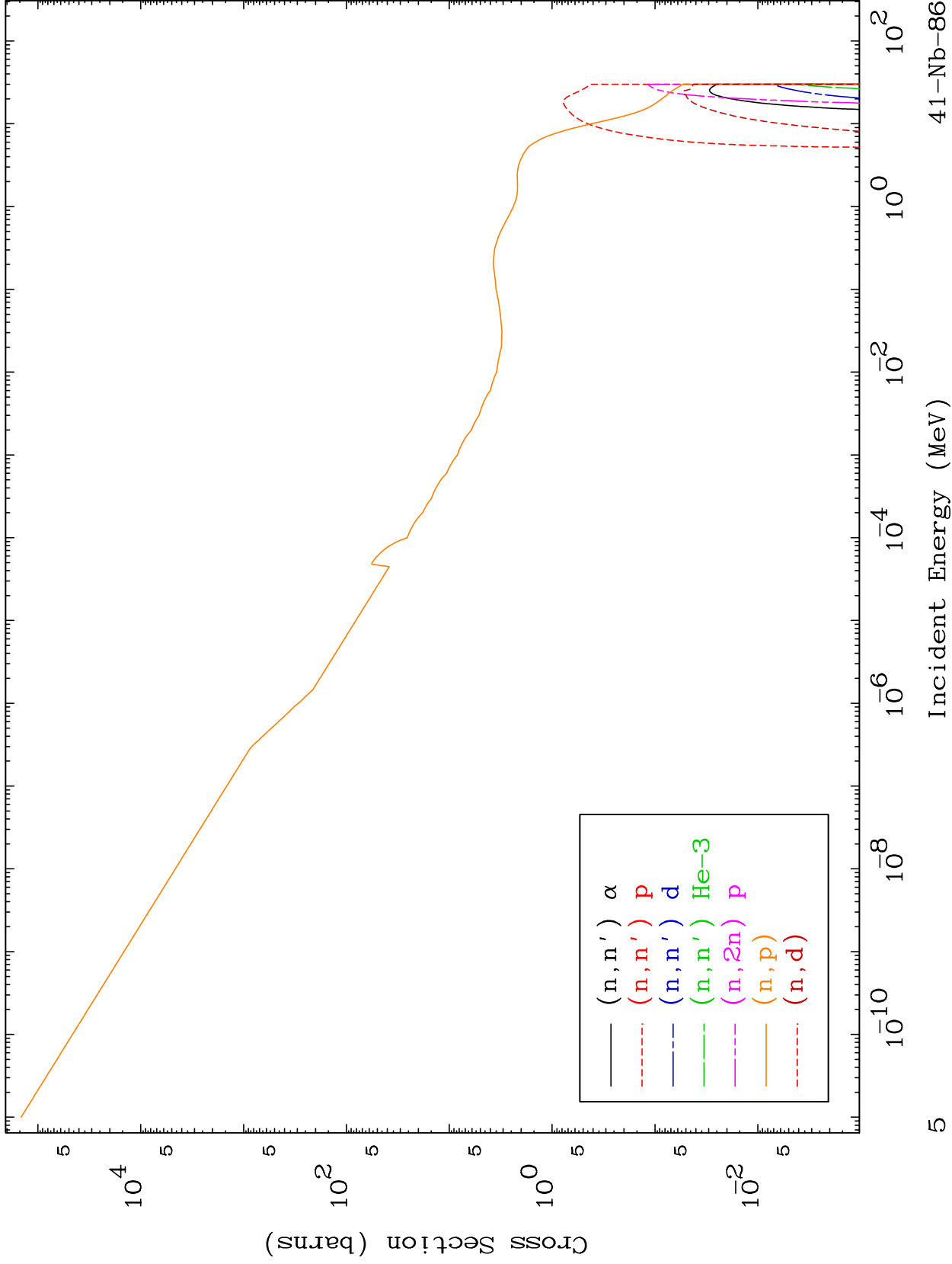
MAT 4104

Neutron Absorption  
293 Kelvin Cross Sections

41-Nb-86



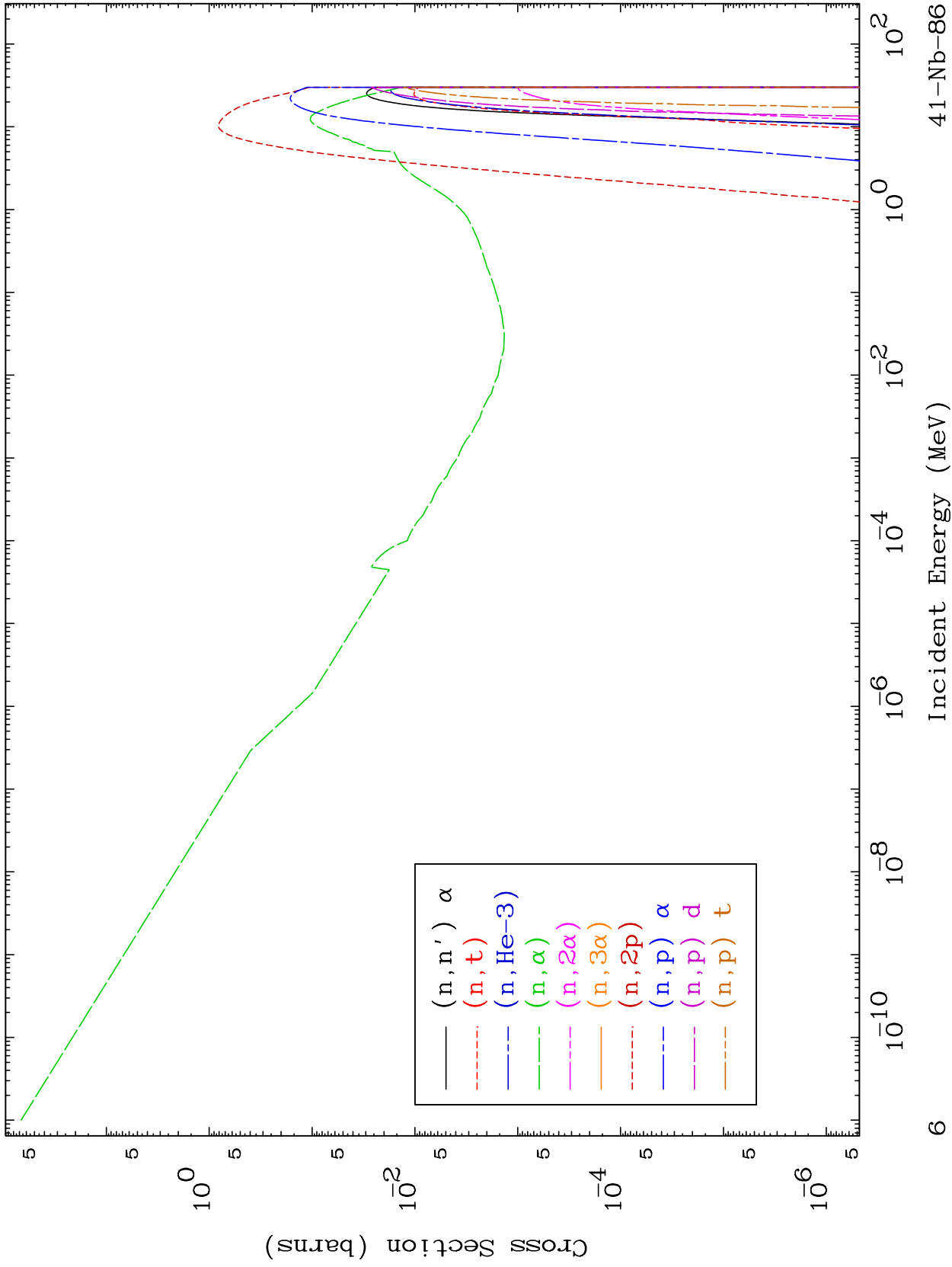




MAT 4104

Charged Particle  
293 Kelvin Cross Sections

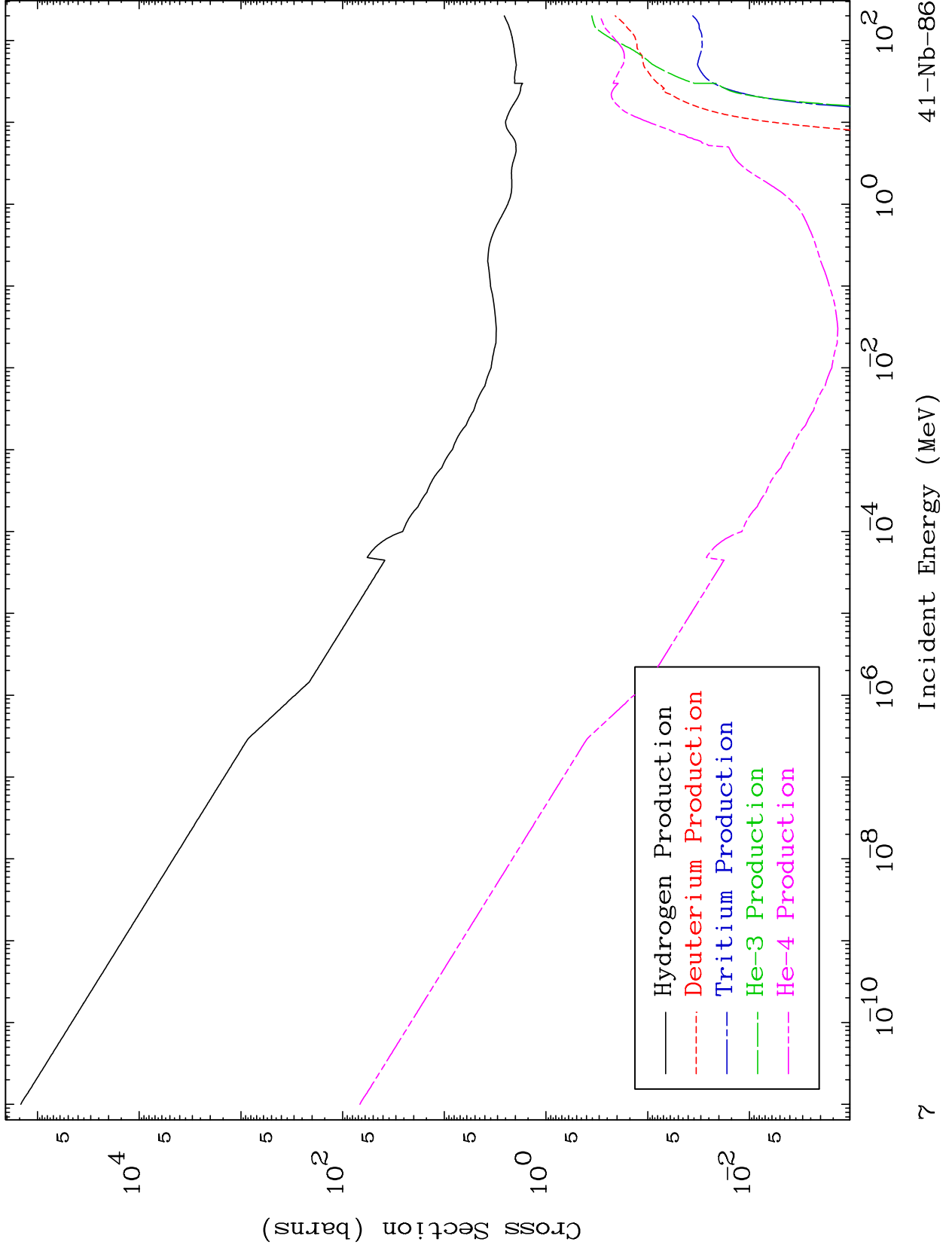
41-Nb-86

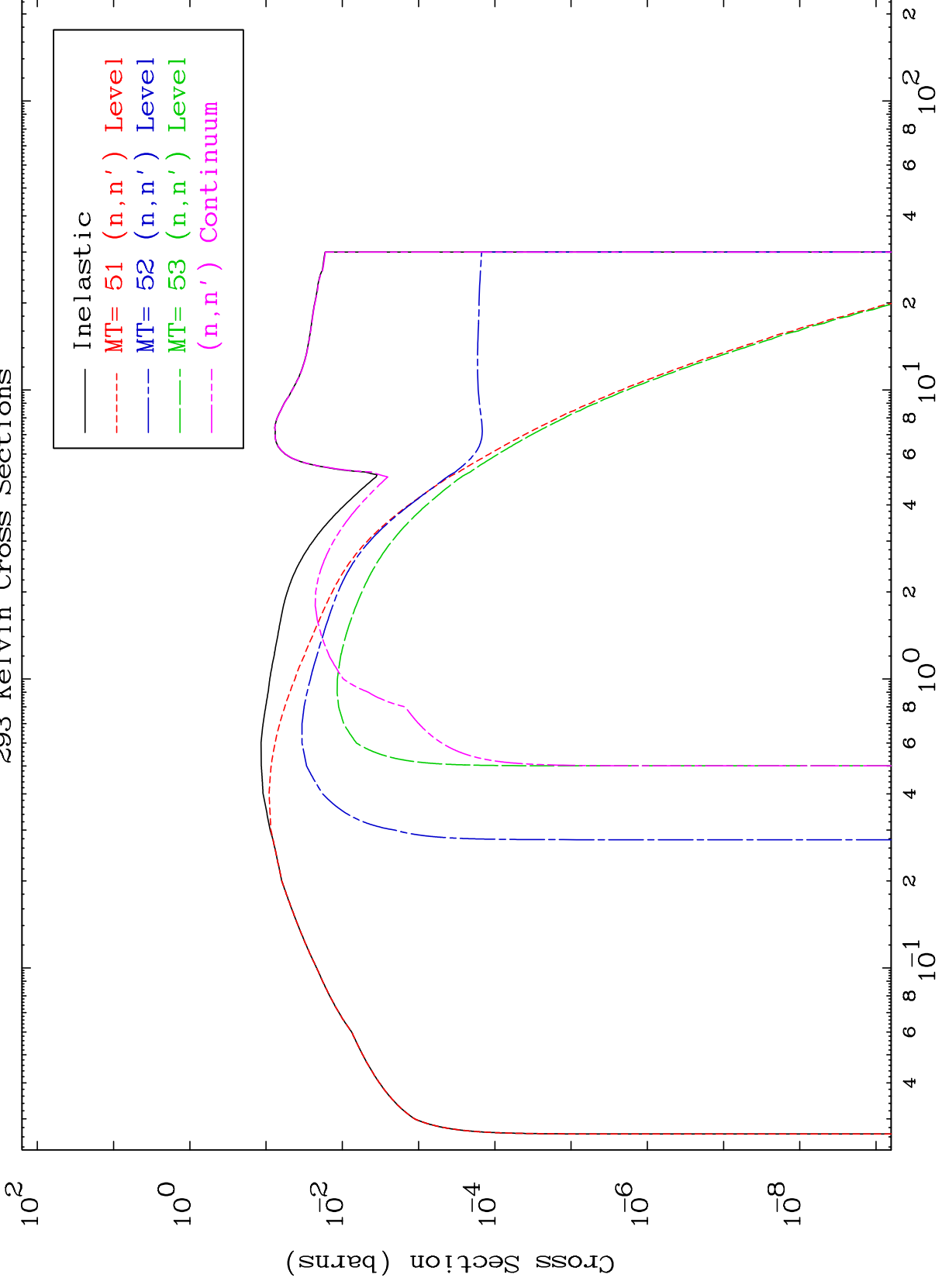


MAT 4104

Particle Production  
293 Kelvin Cross Sections

41-Nb-86

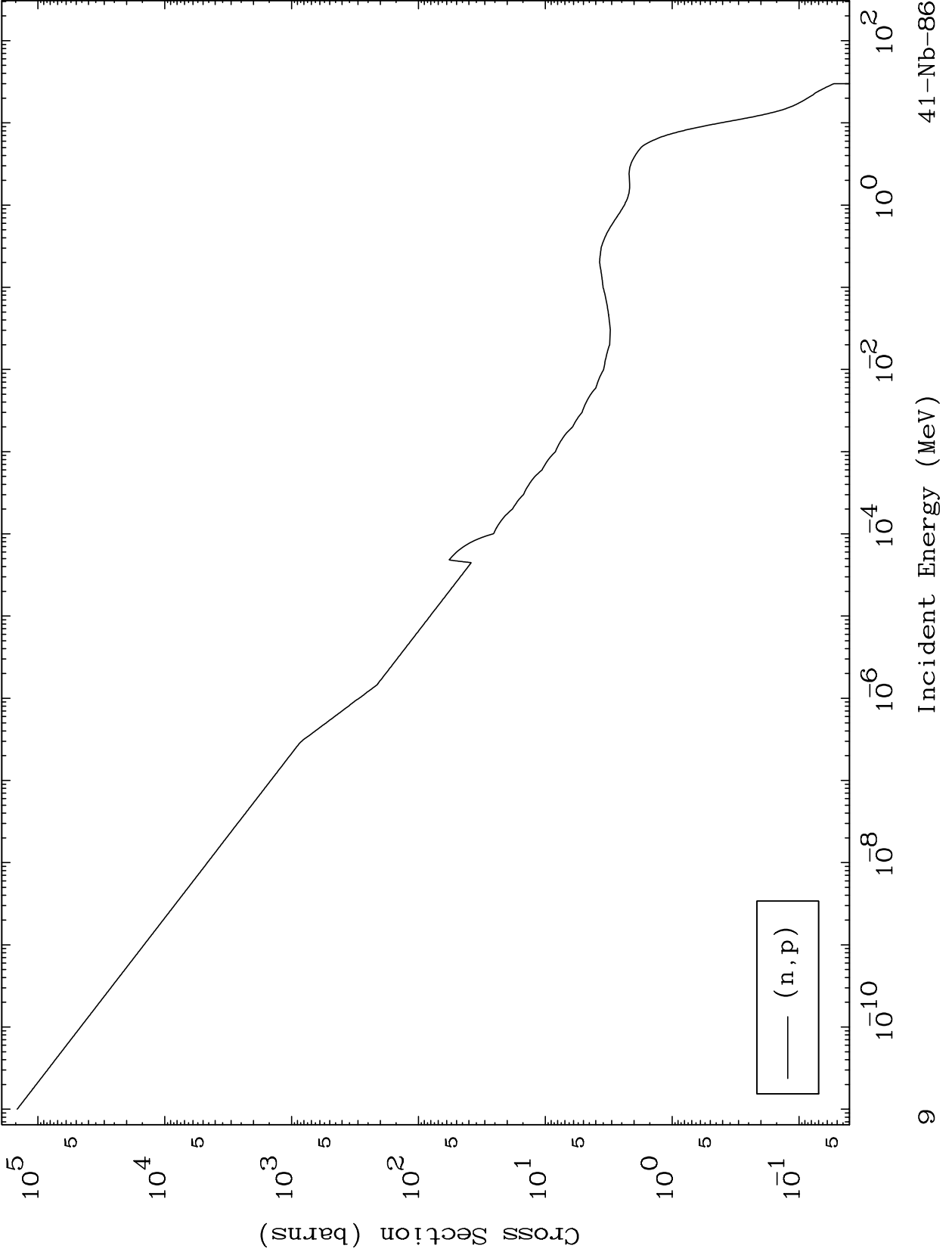




MAT 4104

(n,p) Levels  
293 Kelvin Cross Sections

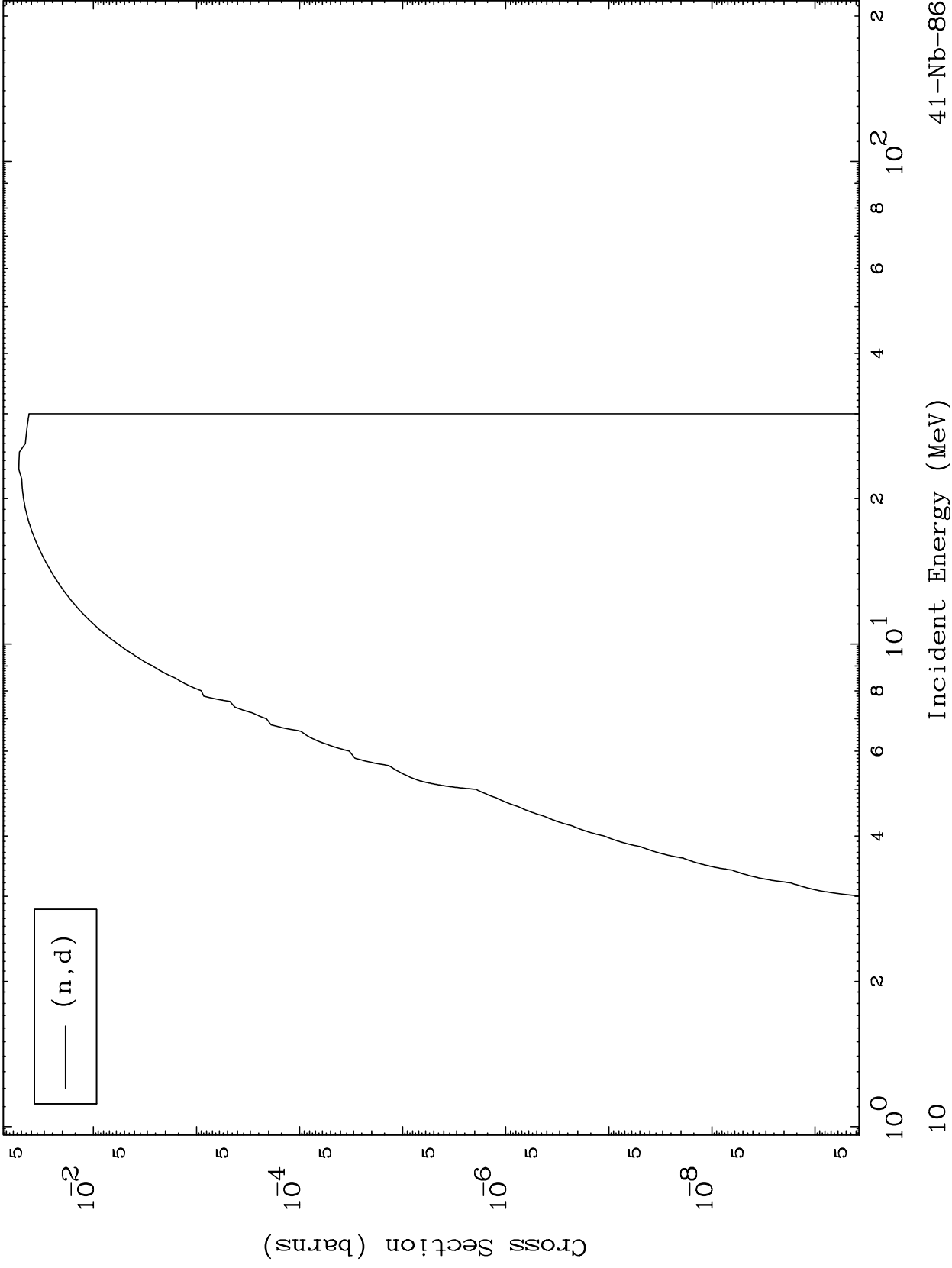
41-Nb-86



MAT 4104

(n,d) Levels  
293 Kelvin Cross Sections

41-Nb-86

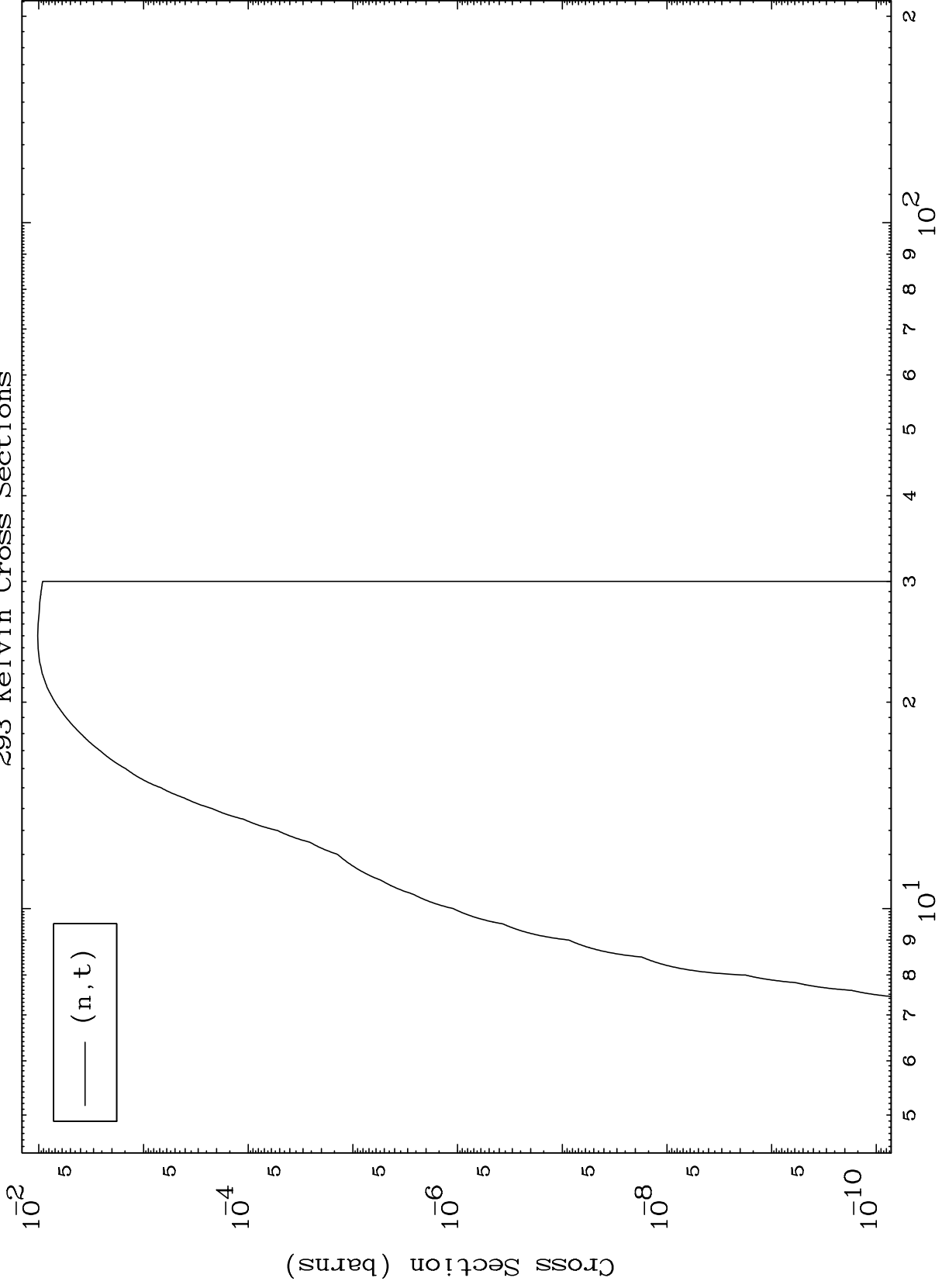


41-Nb-86

MAT 4104

(n,t) Levels  
293 Kelvin Cross Sections

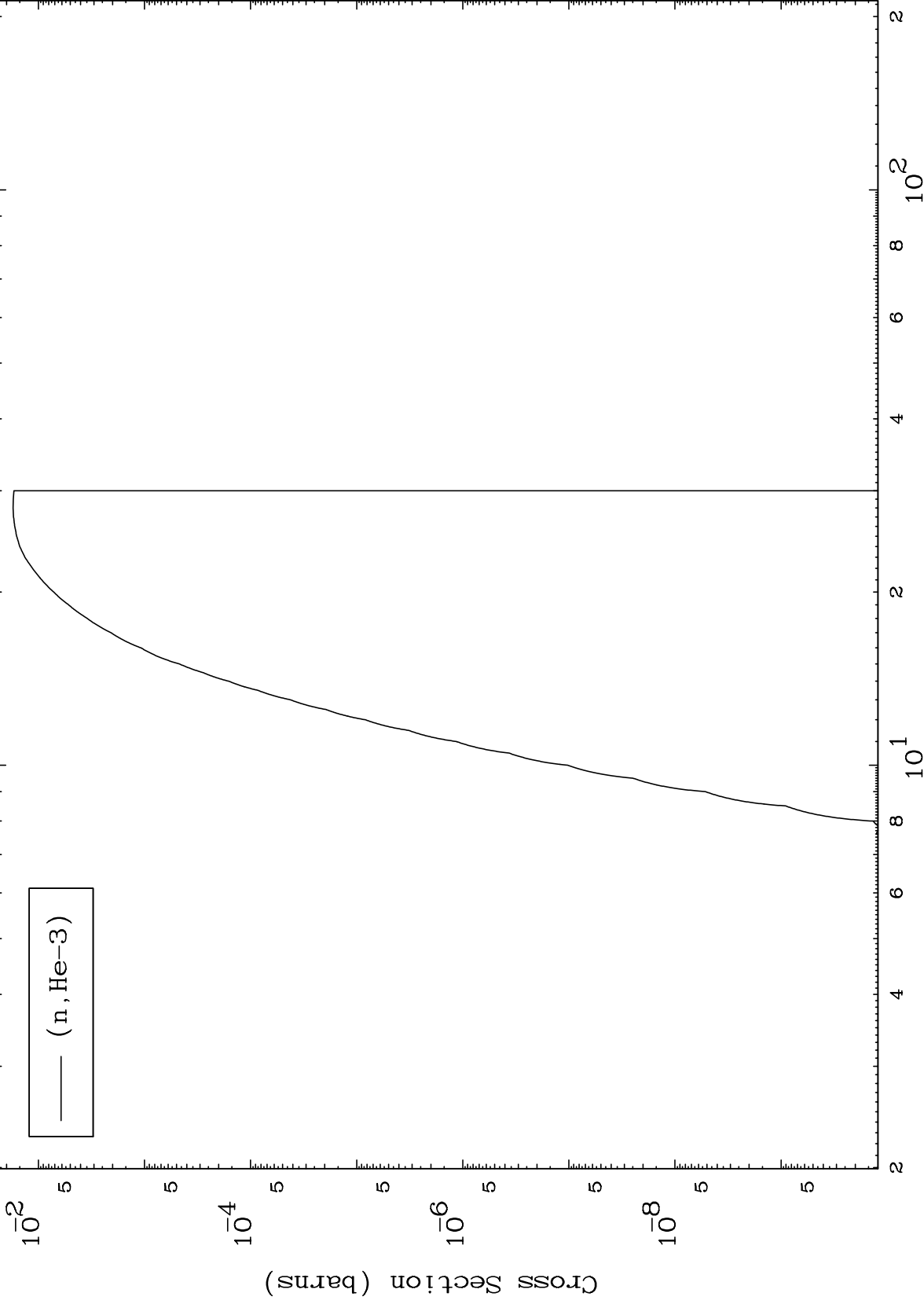
41-Nb-86



MAT 4104

(n,He3) Levels  
293 Kelvin Cross Sections

41-Nb-86



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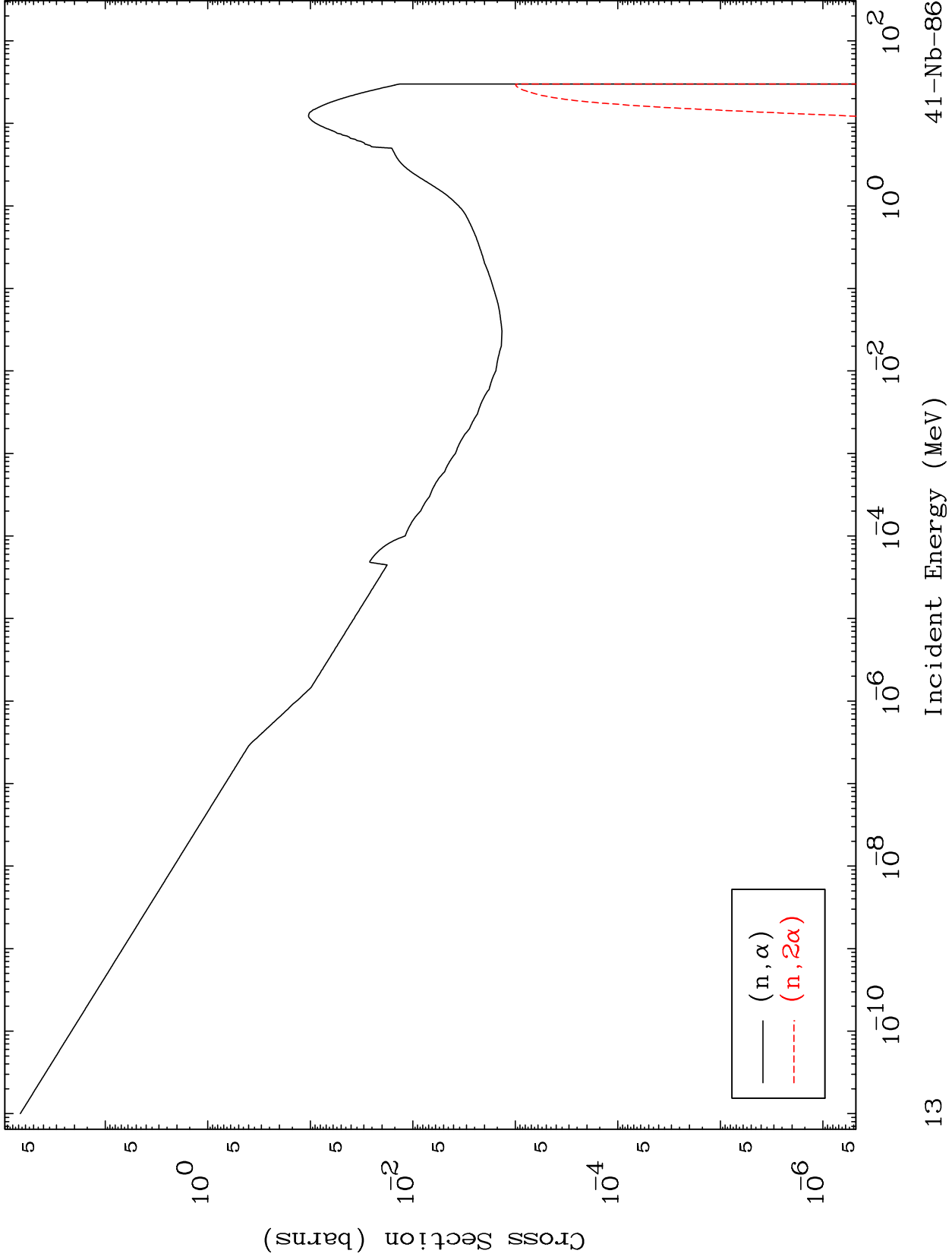
41-Nb-86

Incident Energy (MeV)

MAT 4104

(n,α) Levels  
293 Kelvin Cross Sections

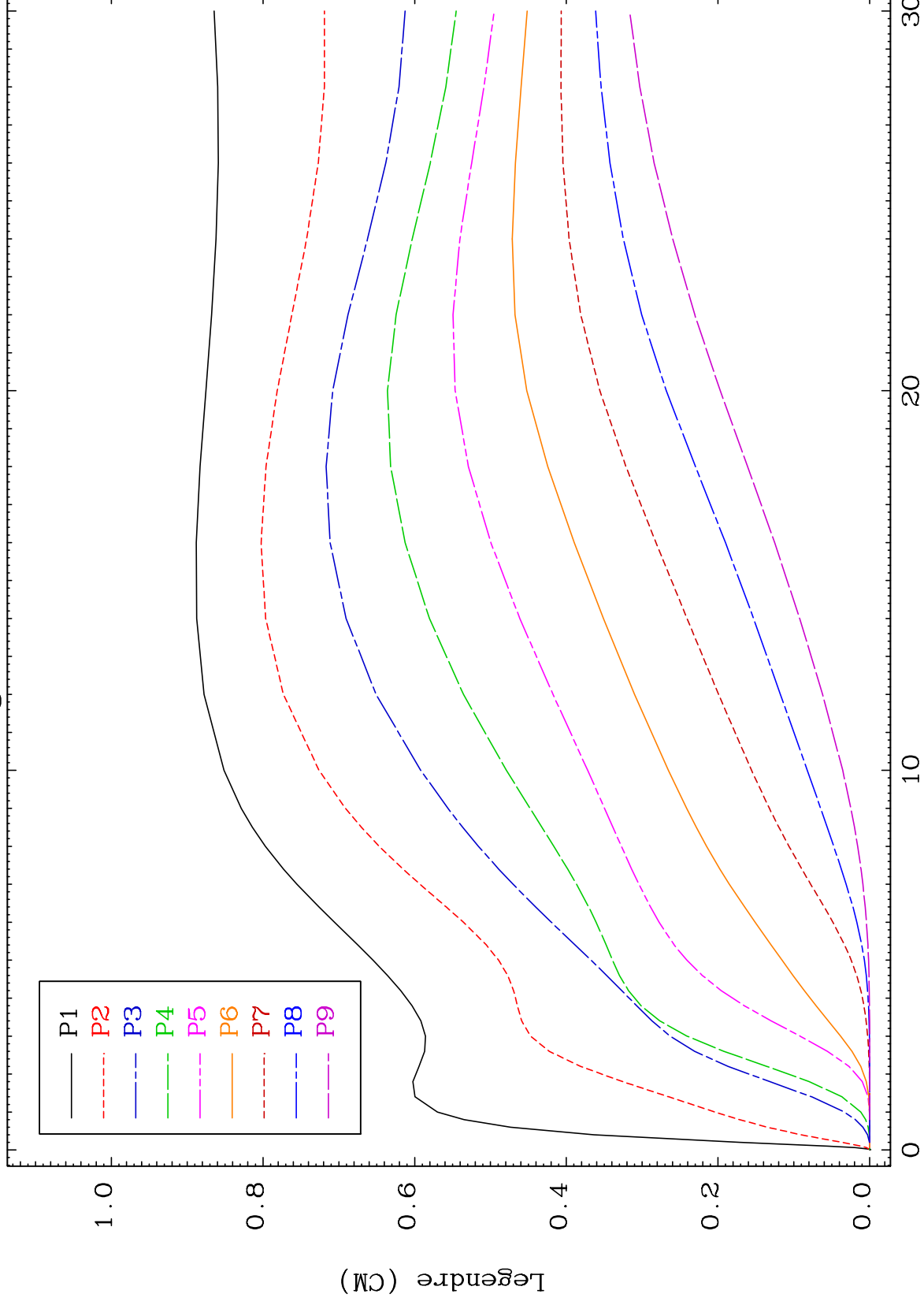
41-Nb-86



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Elastic Legendre Coefficients

41-Nb-86



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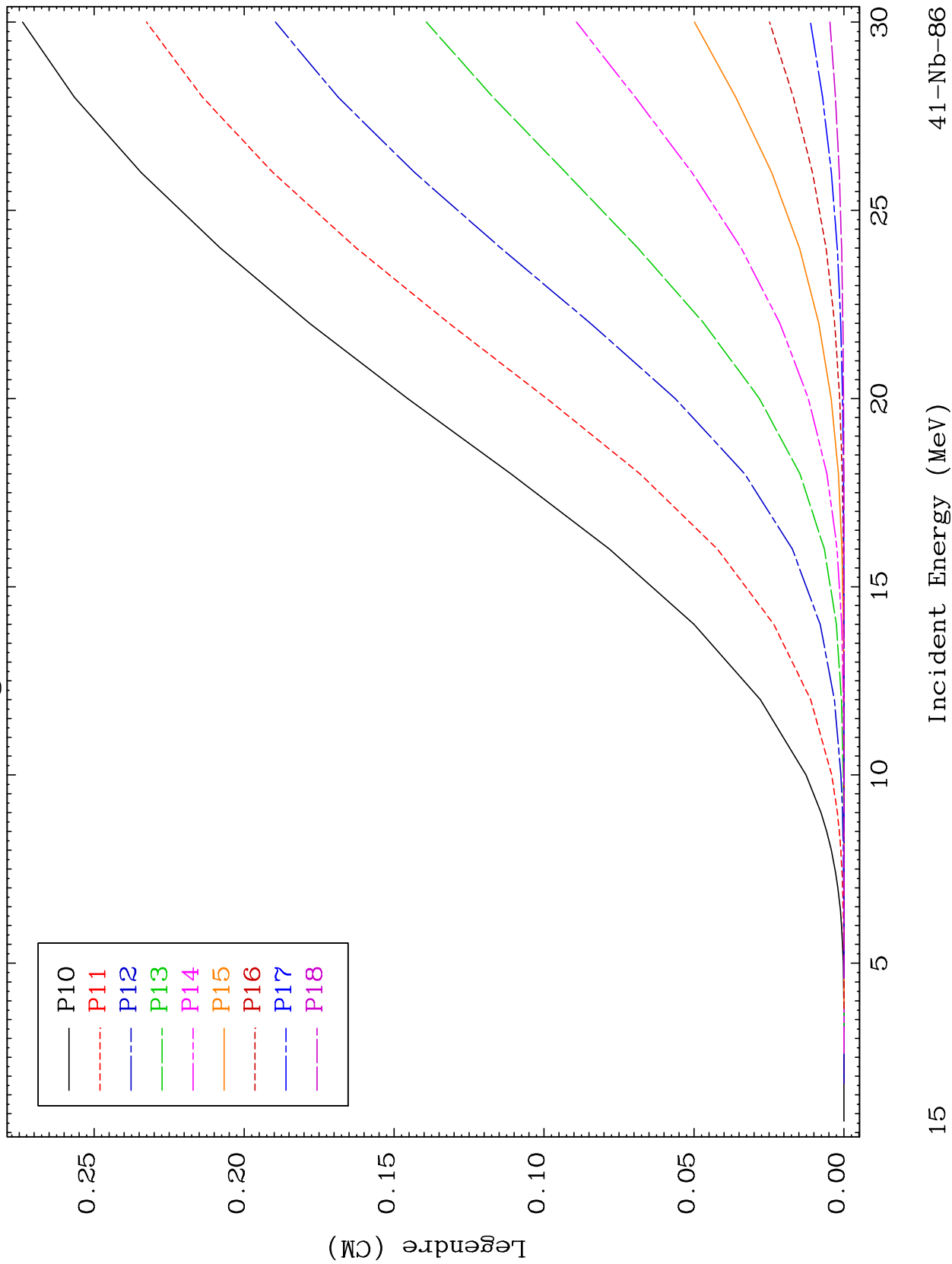
Incident Energy (MeV)

41-Nb-86

MAT 4104

### Elastic Legendre Coefficients

41-Nb-86



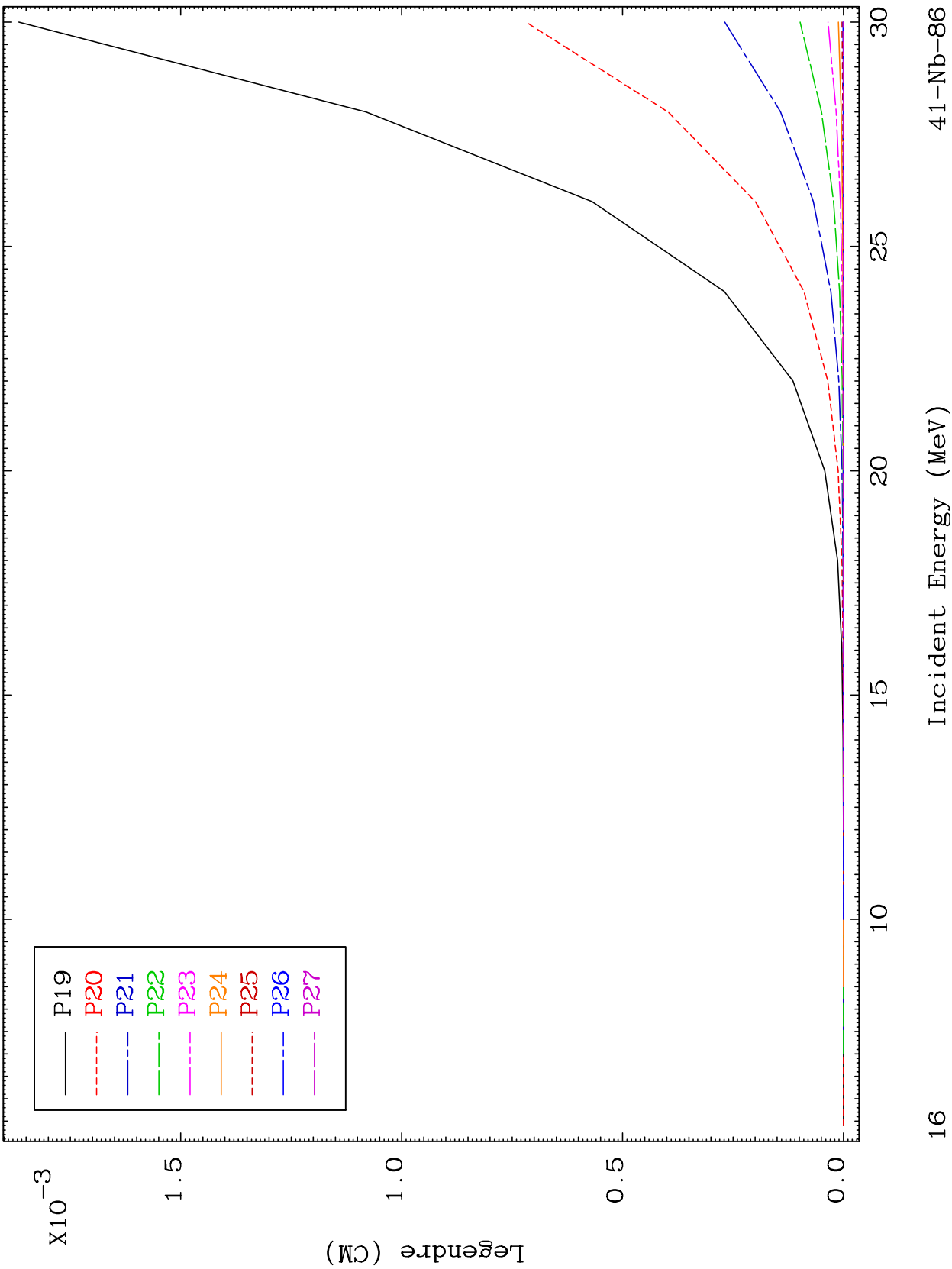
15

41-Nb-86

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Elastic Legendre Coefficients

41-Nb-86



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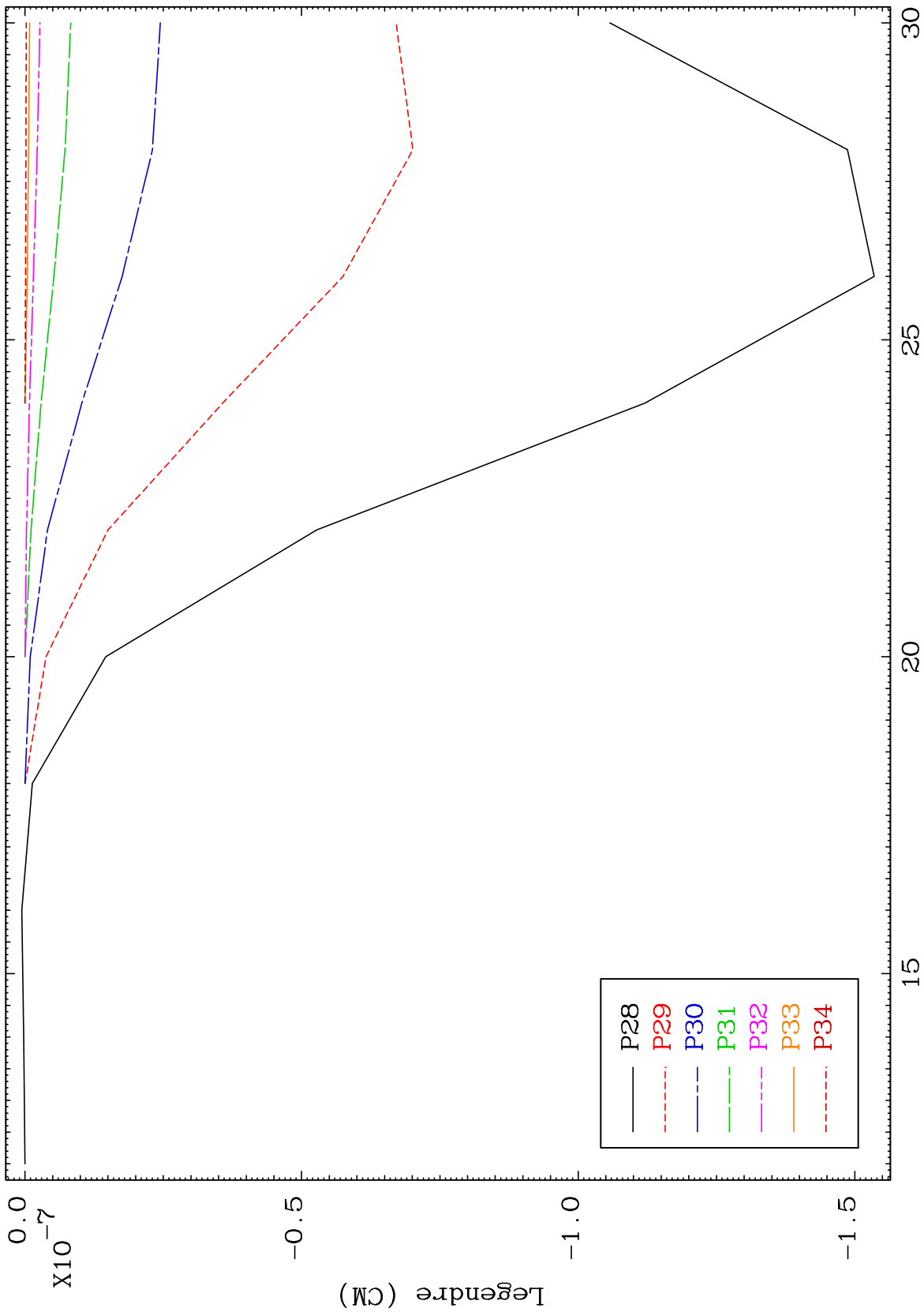
Incident Energy (MeV)

41-Nb-86

MAT 4104

41-Nb-86

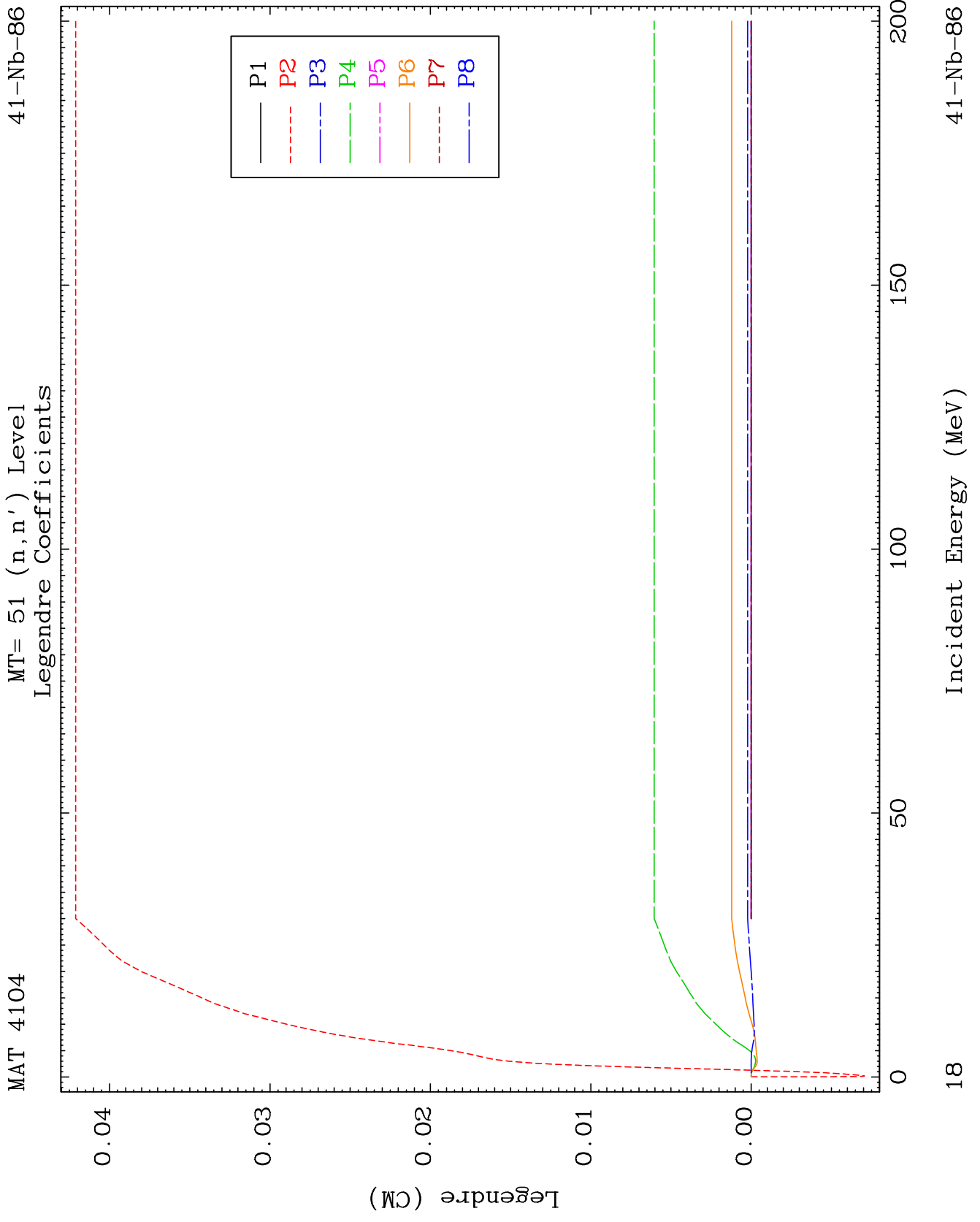
Elastic Legendre Coefficients



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Incident Energy (MeV)

41-Nb-86

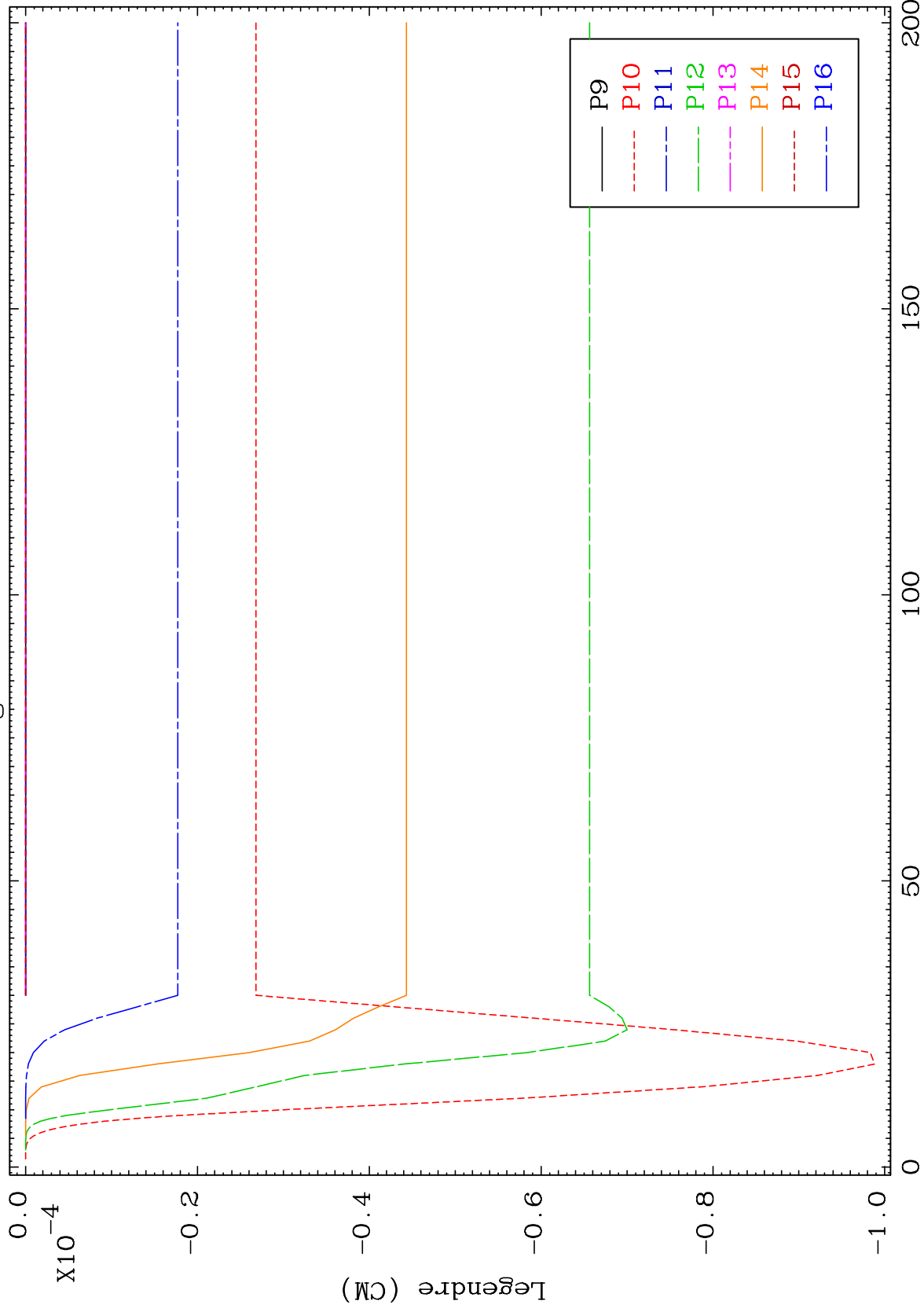


MAT 4104

MT= 51 (n,n') Level

41-Nb-86

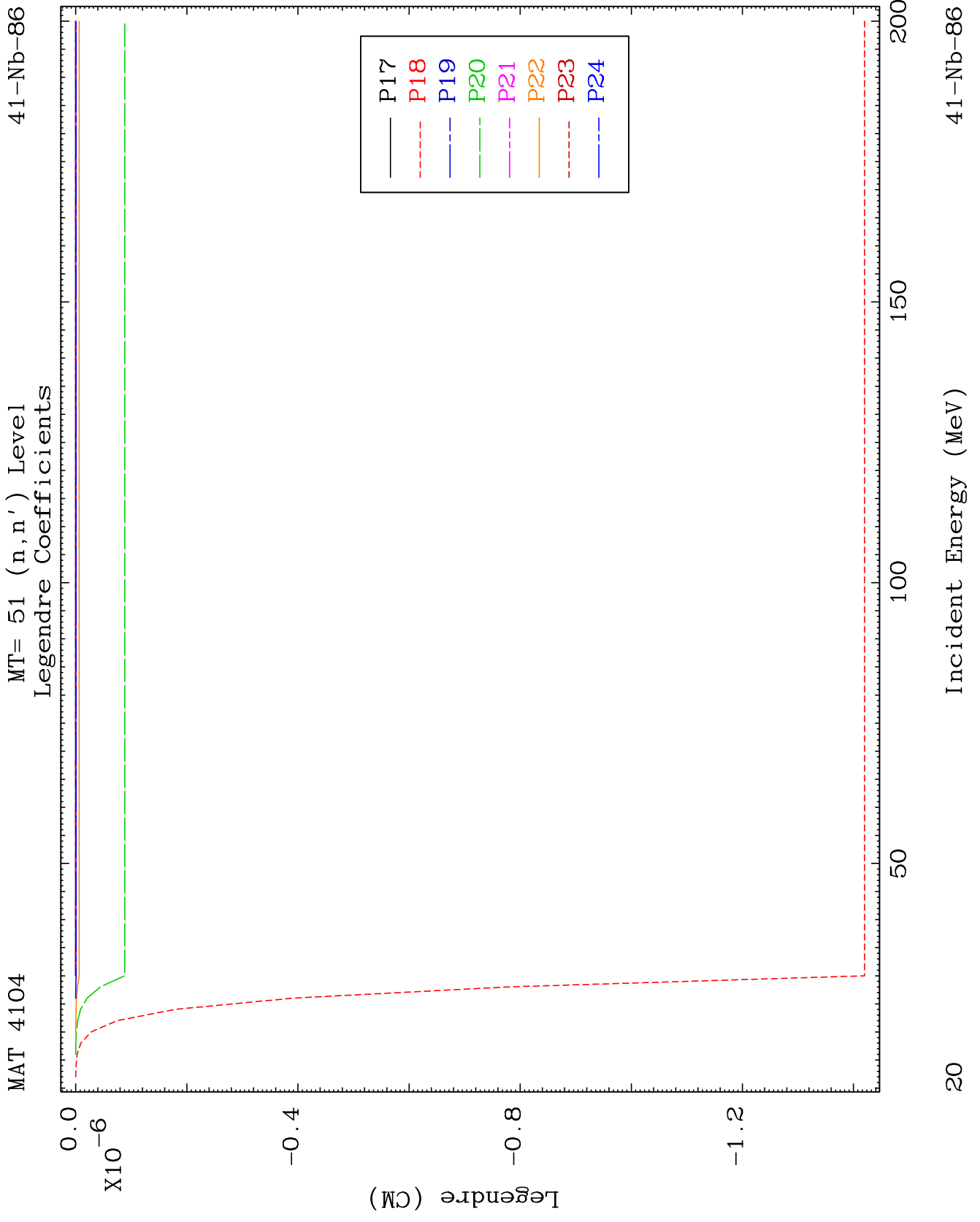
Legendre Coefficients



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Incident Energy (MeV)

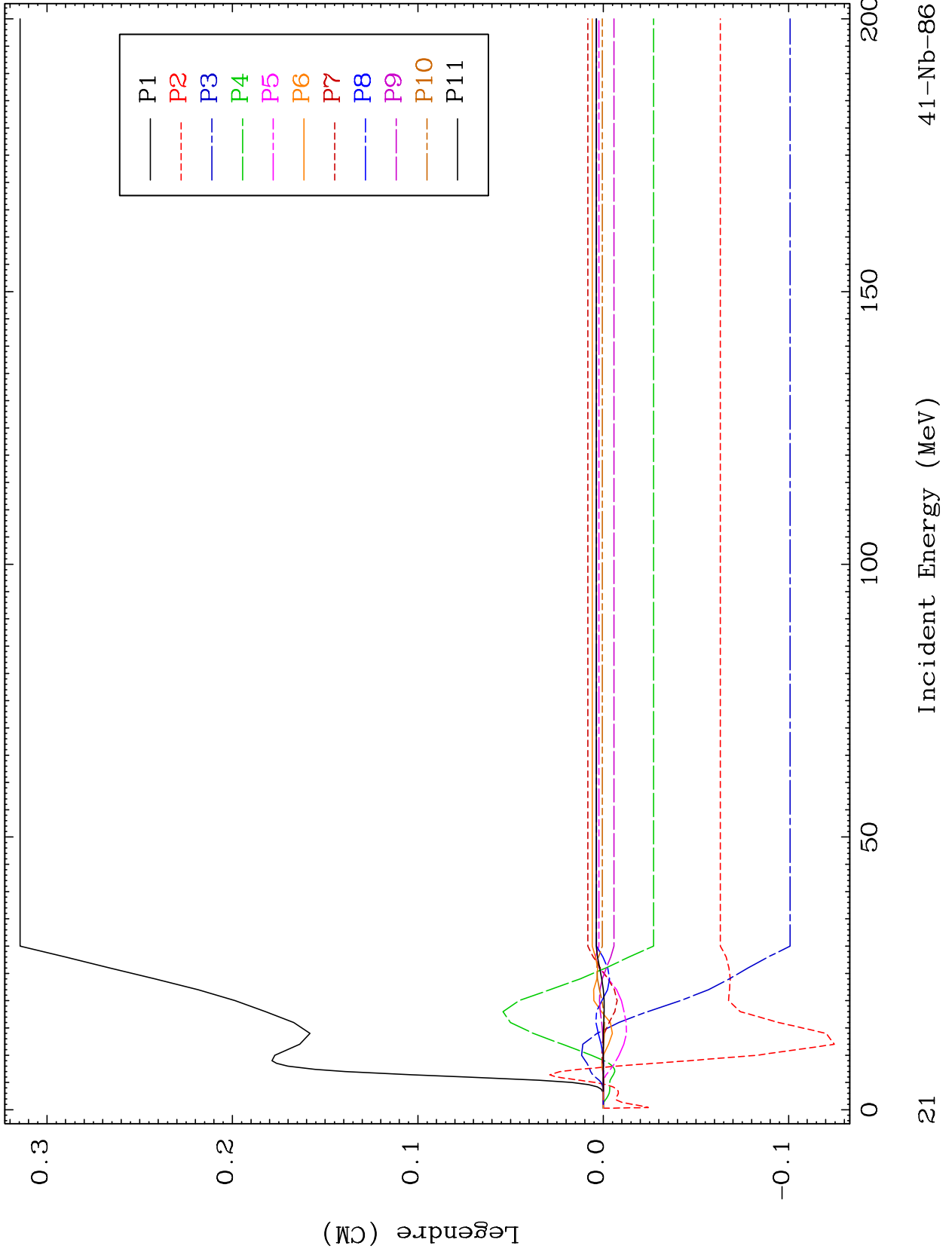
41-Nb-86



MAT 4104

MT= 52 (n,n') Level  
Legendre Coefficients

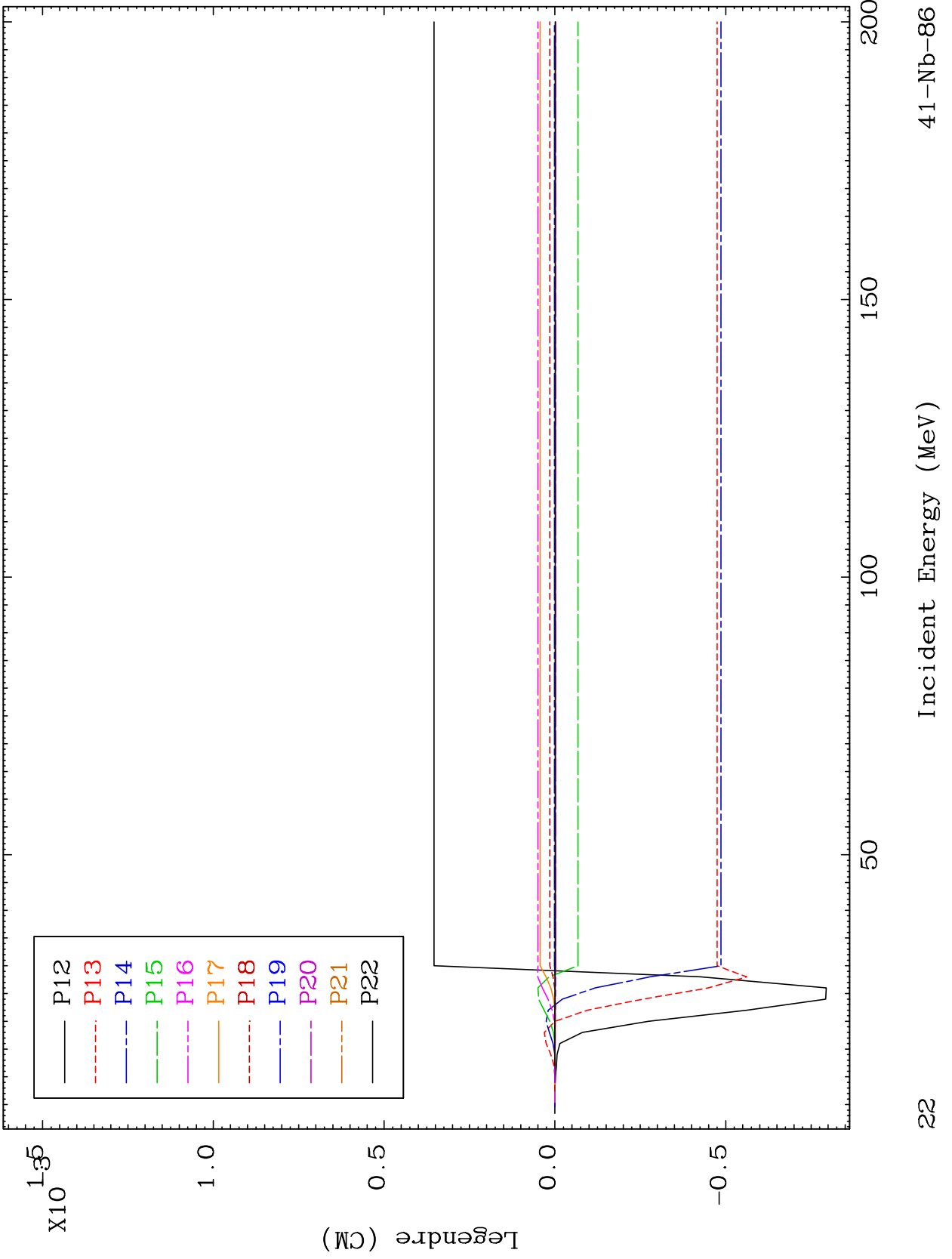
41-Nb-86



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Incident Energy (MeV)

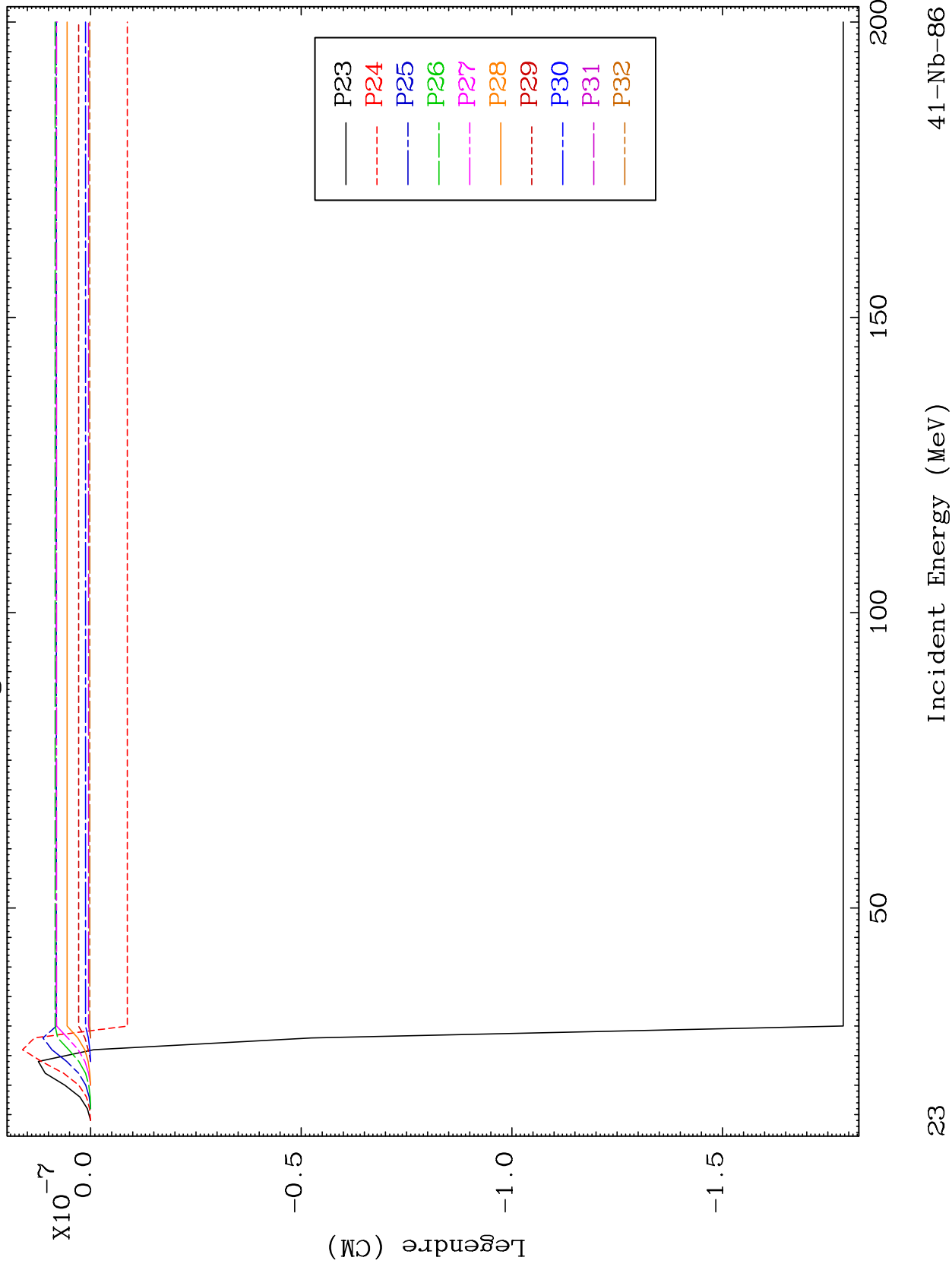
41-Nb-86



MAT 4104

MT= 52 (n,n') Level  
Legendre Coefficients

41-Nb-86



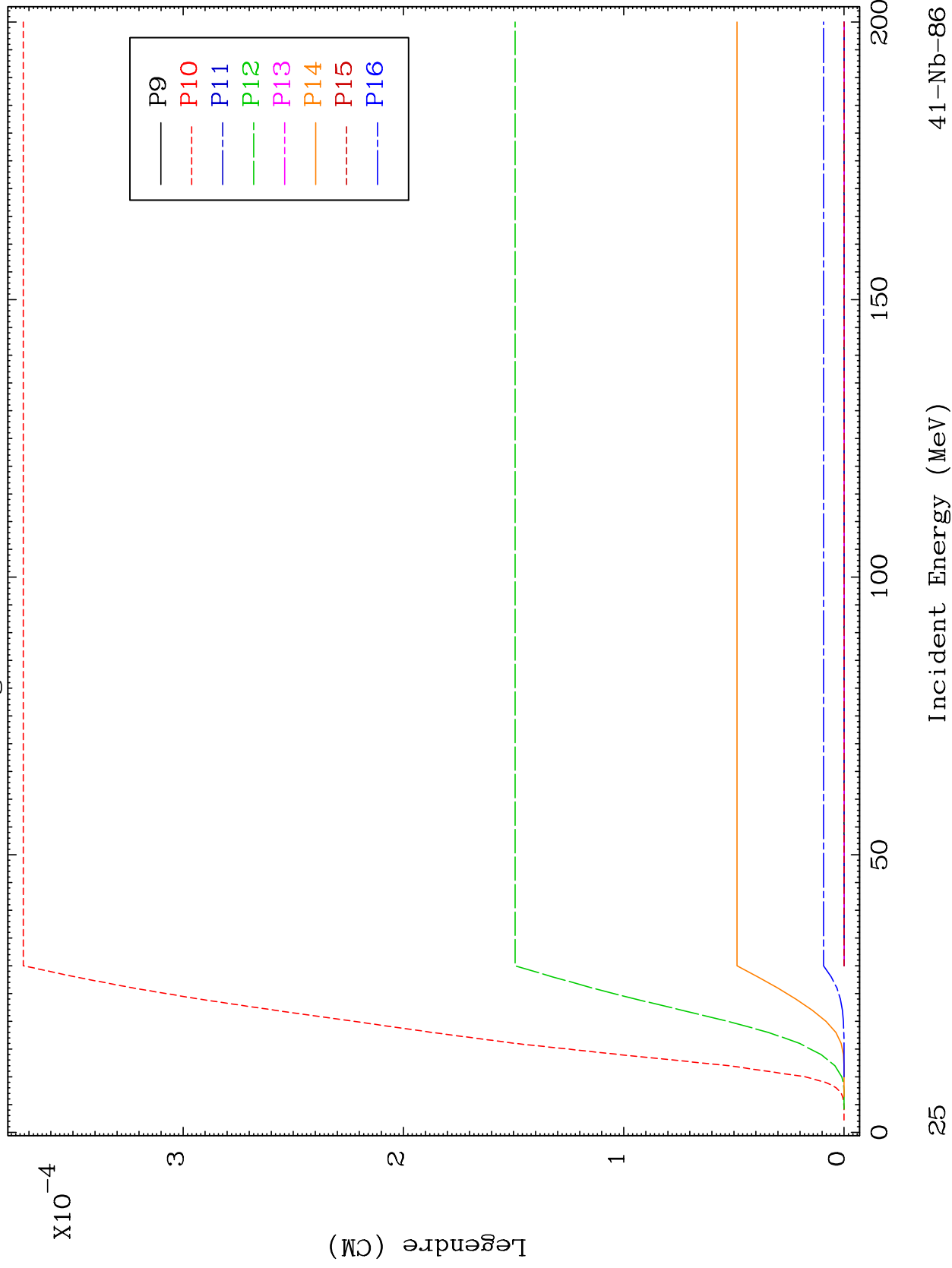
23



MAT 4104

MT= 53 (n,n') Level  
Legendre Coefficients

41-Nb-86



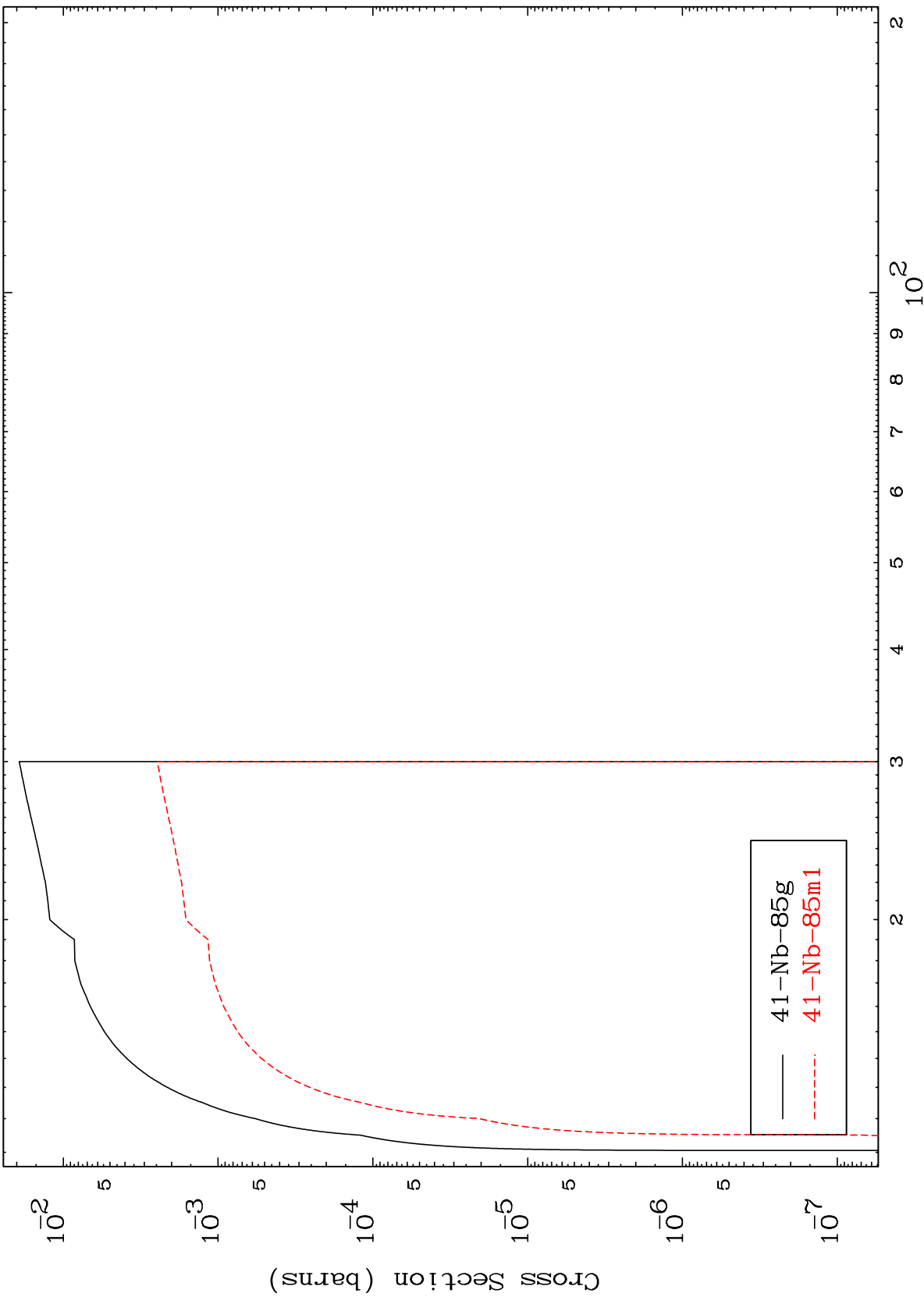
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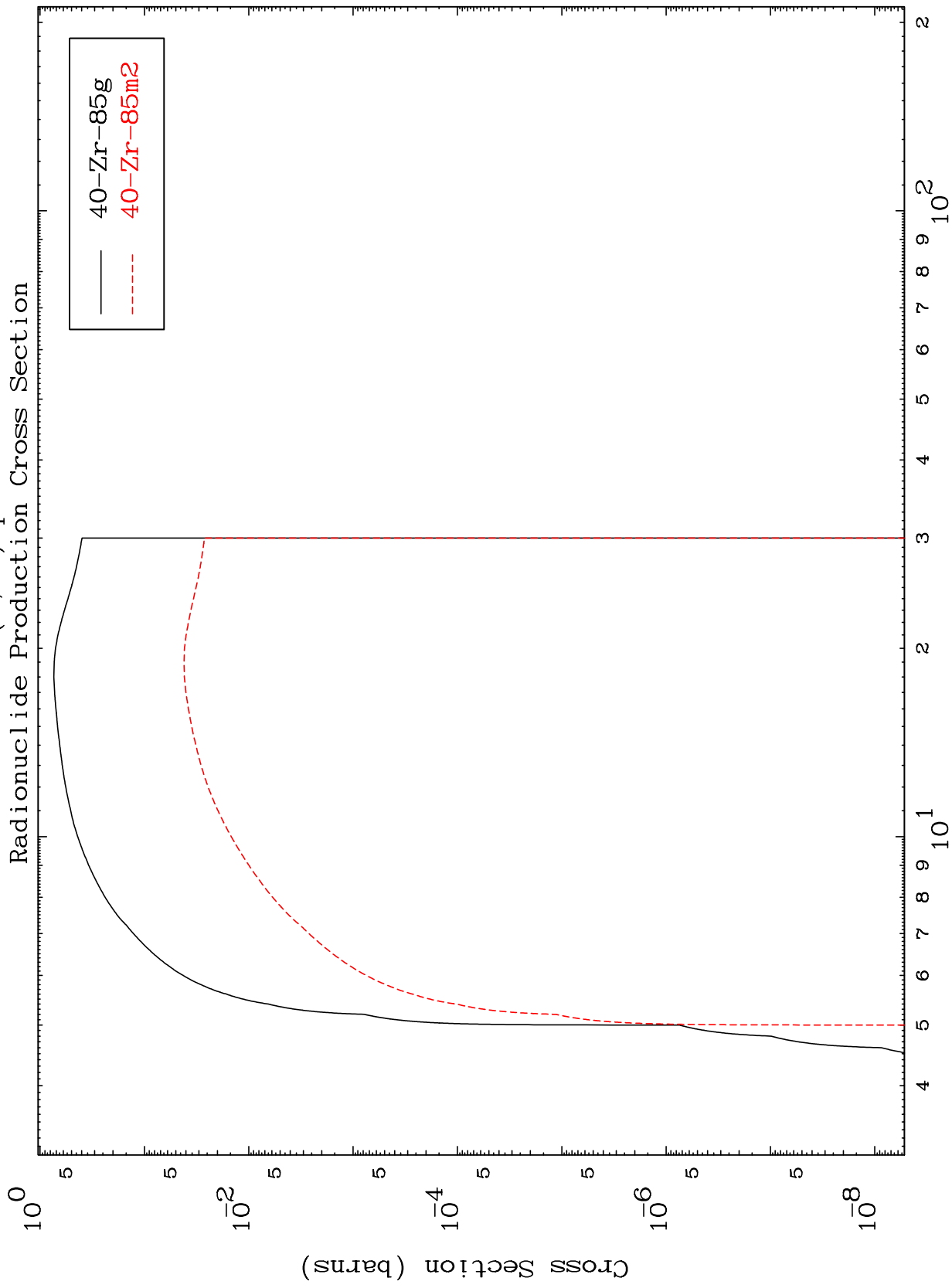
Incident Energy (MeV)

41-Nb-86

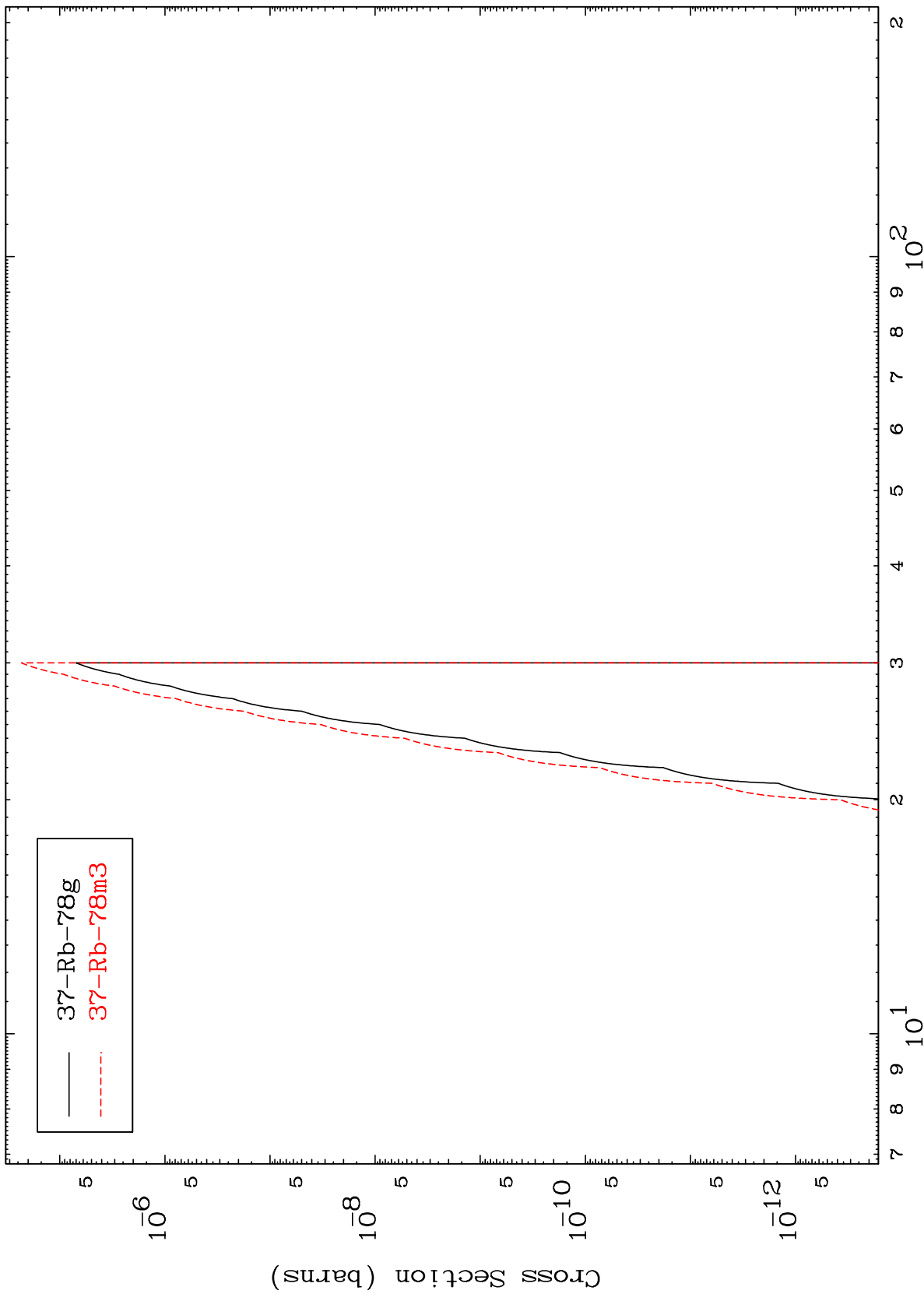


(n,2n)  
Radionuclide Production Cross Section

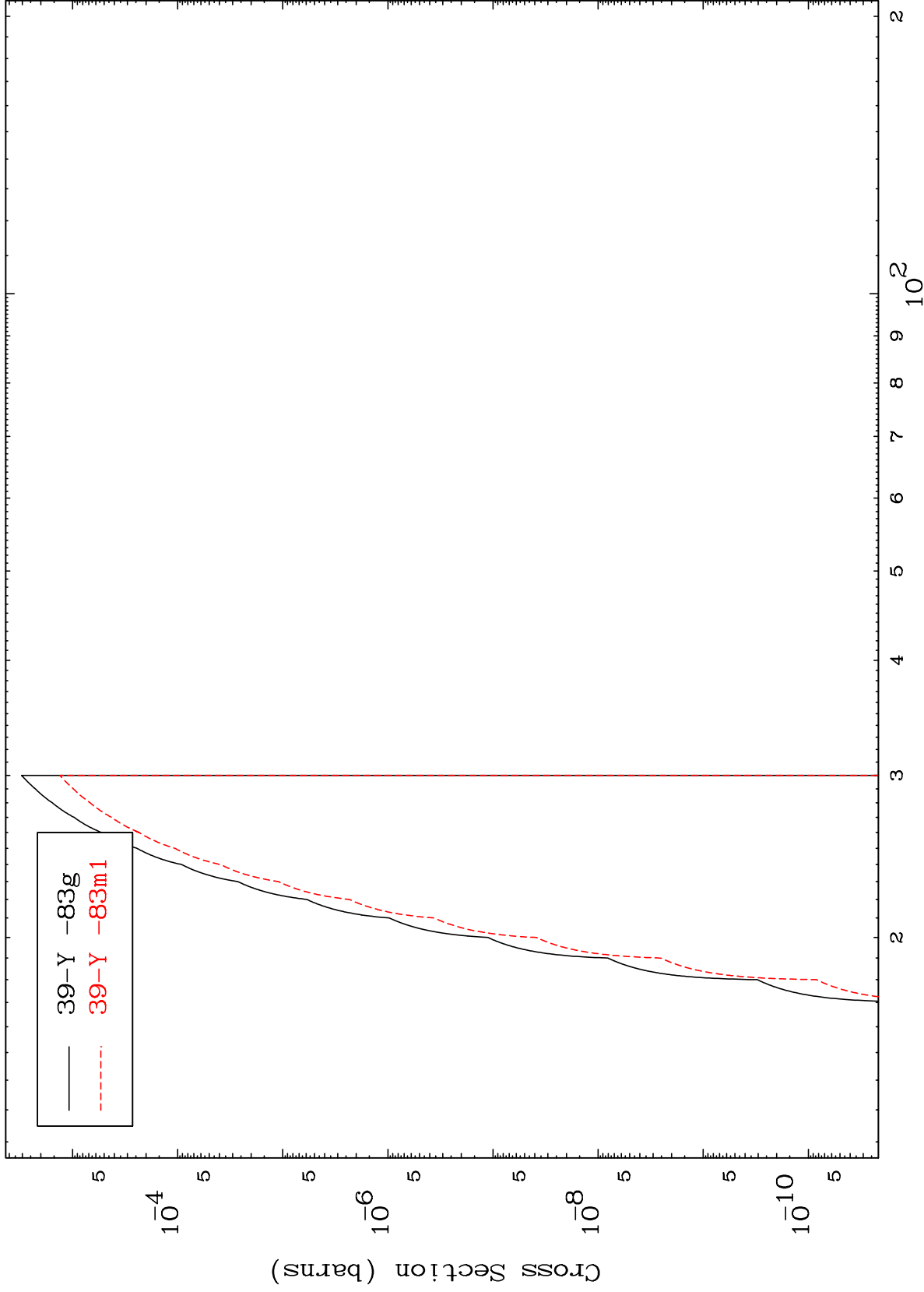




(n,n') 2α  
Radionuclide Production Cross Section



Radionuclide Production Cross Section

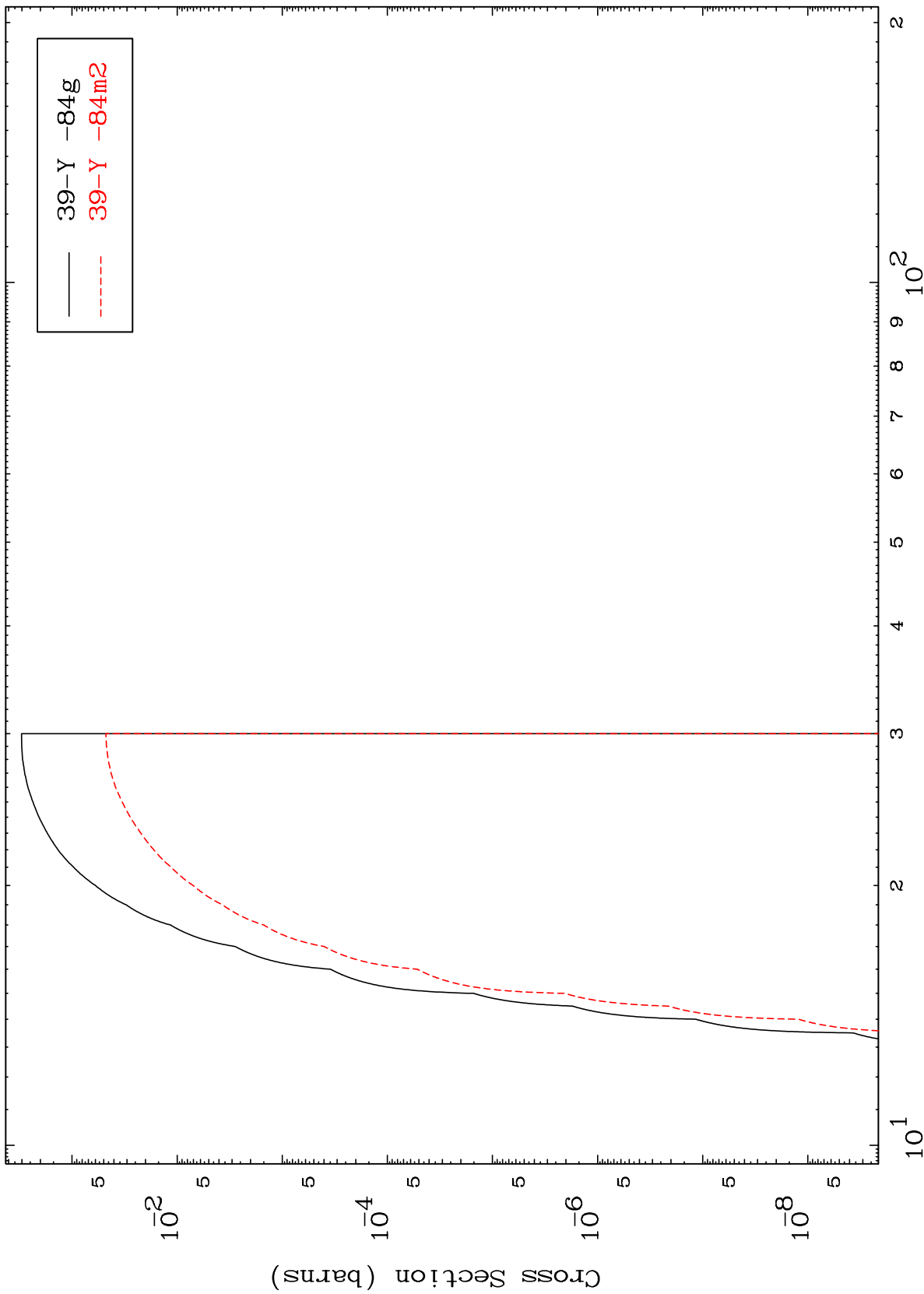


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(n,2n) p

41-Nb-86

Radionuclide Production Cross Section



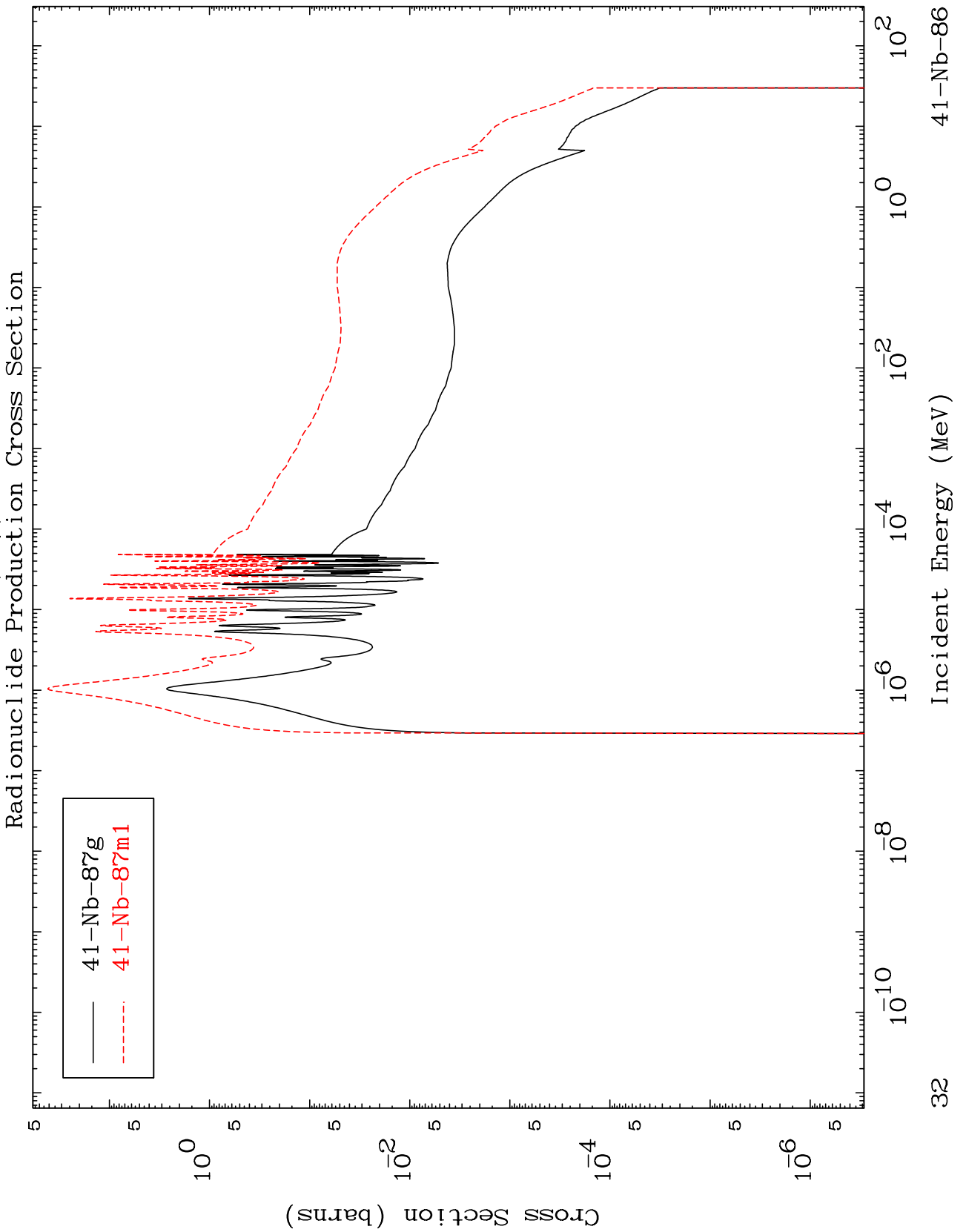
31

Incident Energy (MeV)

41-Nb-86

MAT 4104

41-Nb-86

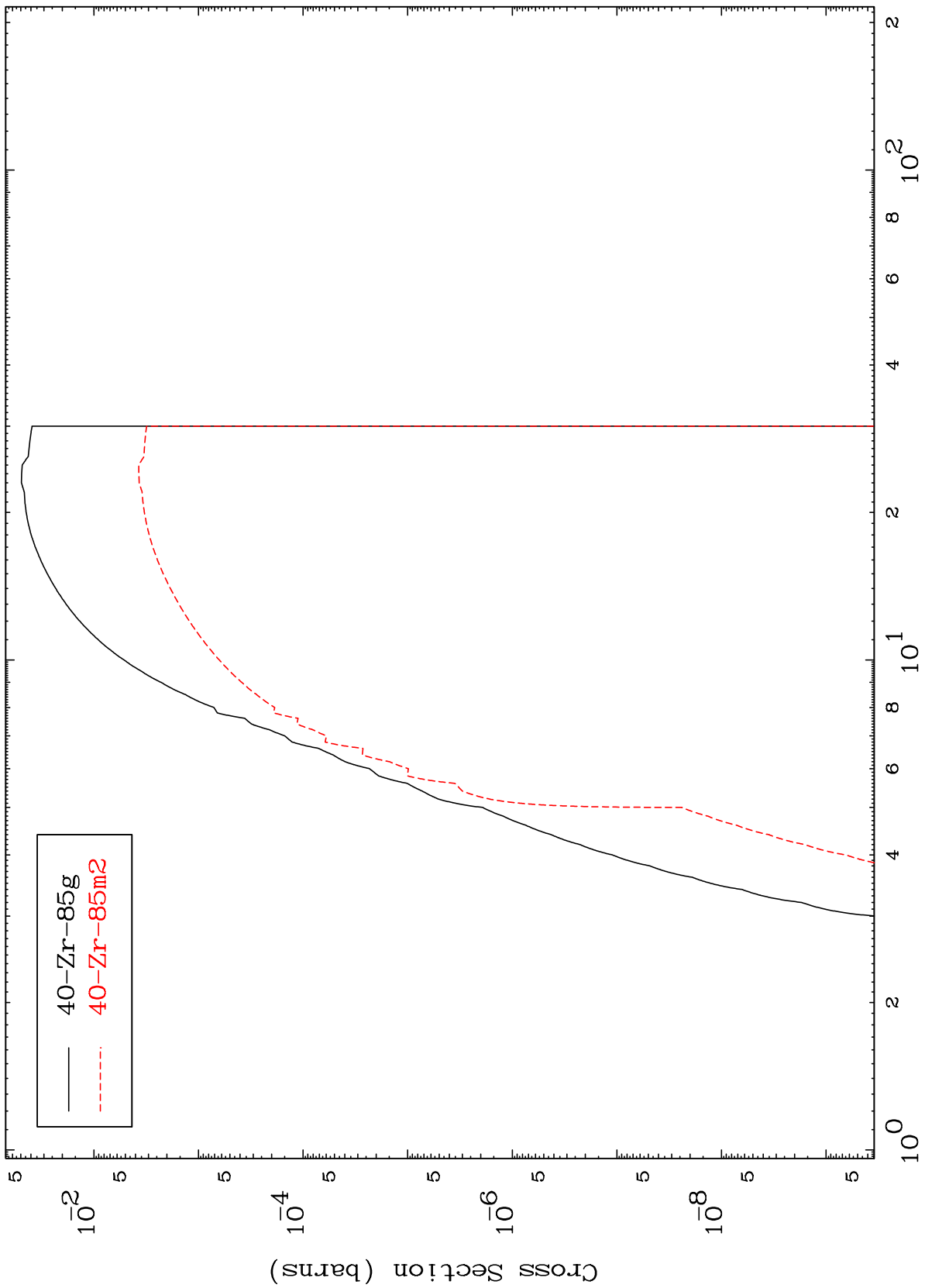


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(n,d)

41-Nb-86

Radionuclide Production Cross Section



33

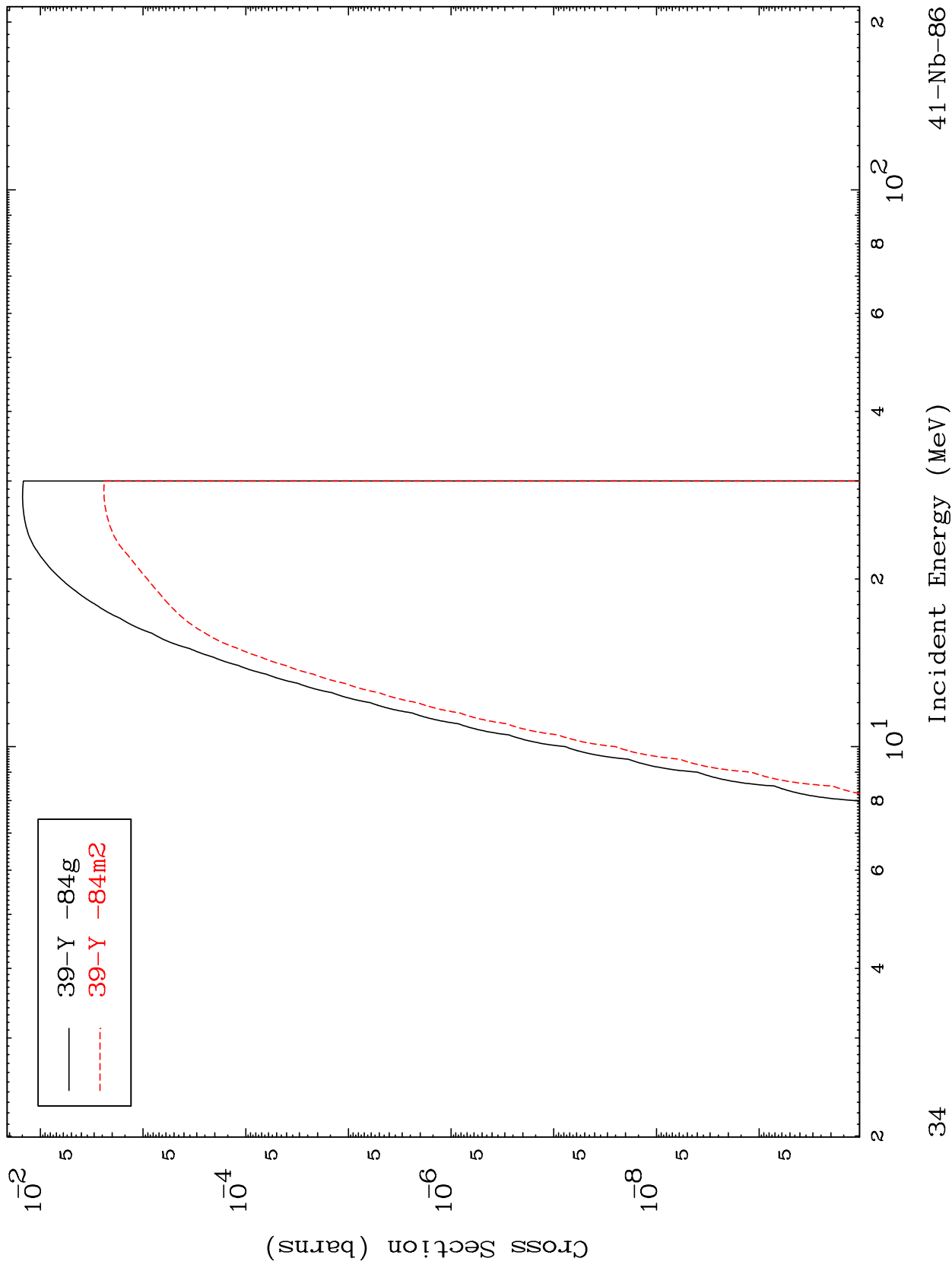
Incident Energy (MeV)

41-Nb-86

MAT 4104

41-Nb-86

Radionuclide Production Cross Section



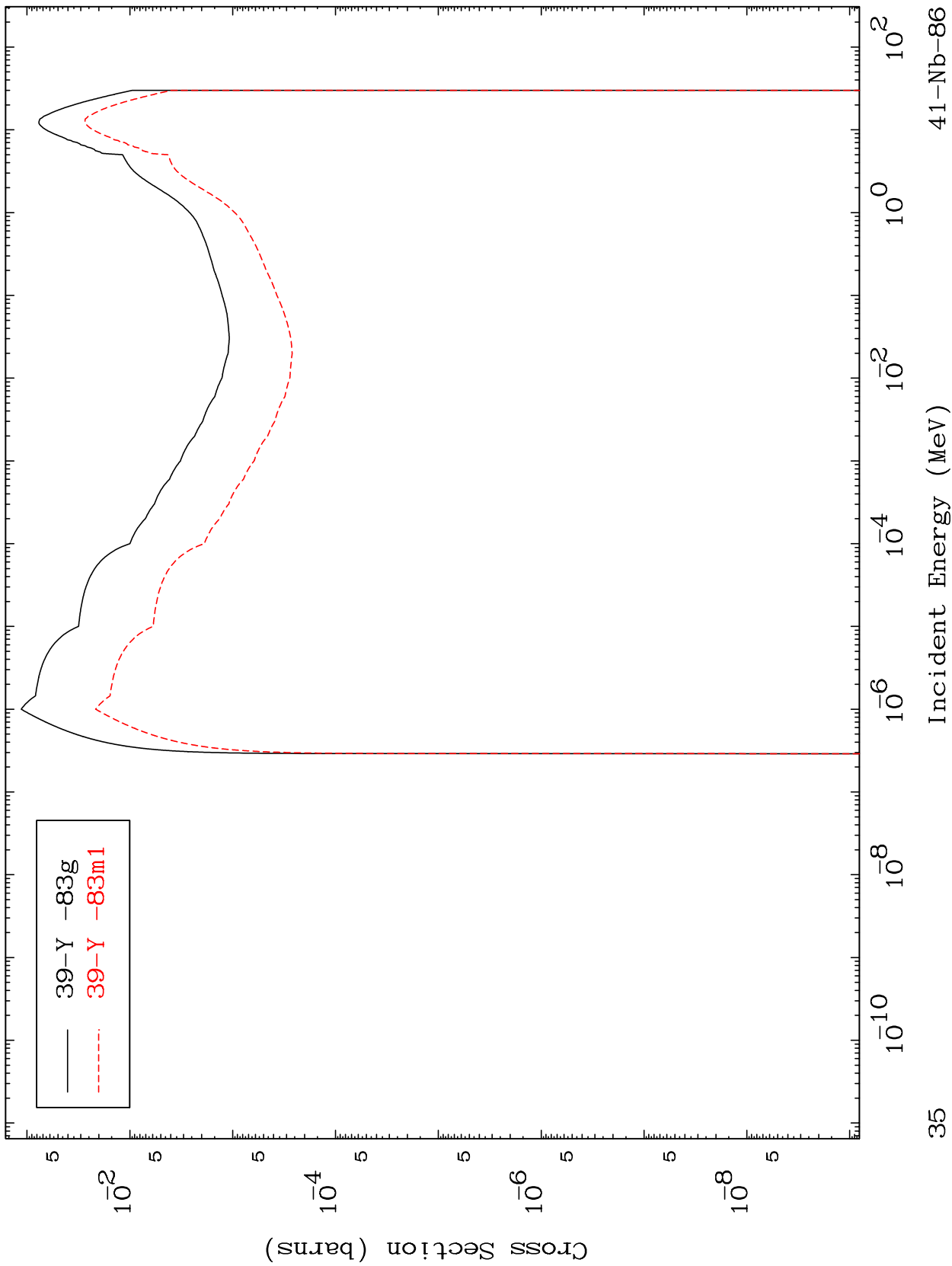
34

41-Nb-86

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41-Nb-86

Radionuclide Production Cross Section



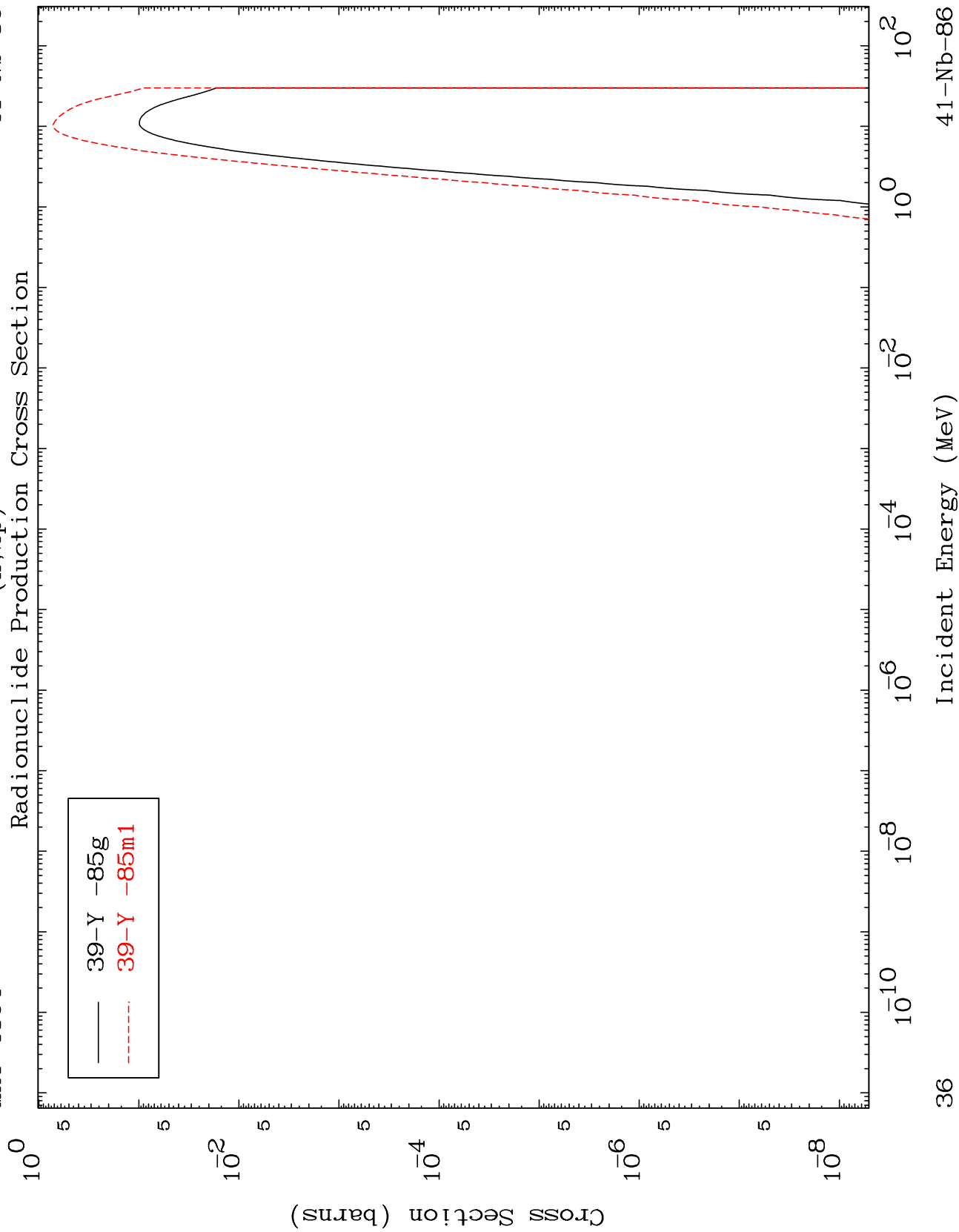
35

41-Nb-86

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41-Nb-86

(n,2p)  
Radionuclide Production Cross Section

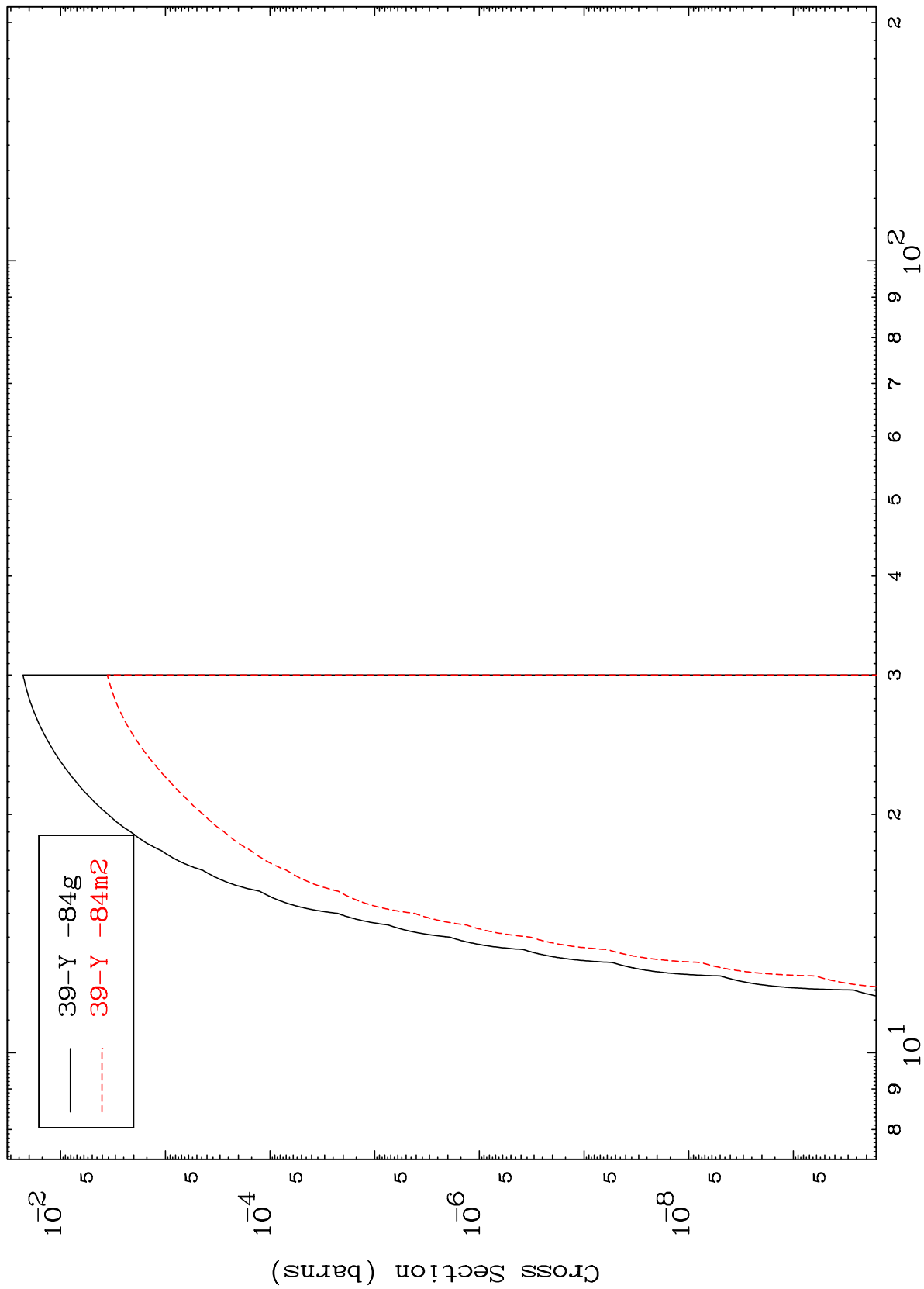


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(n,p) d

41-Nb-86

Radionuclide Production Cross Section



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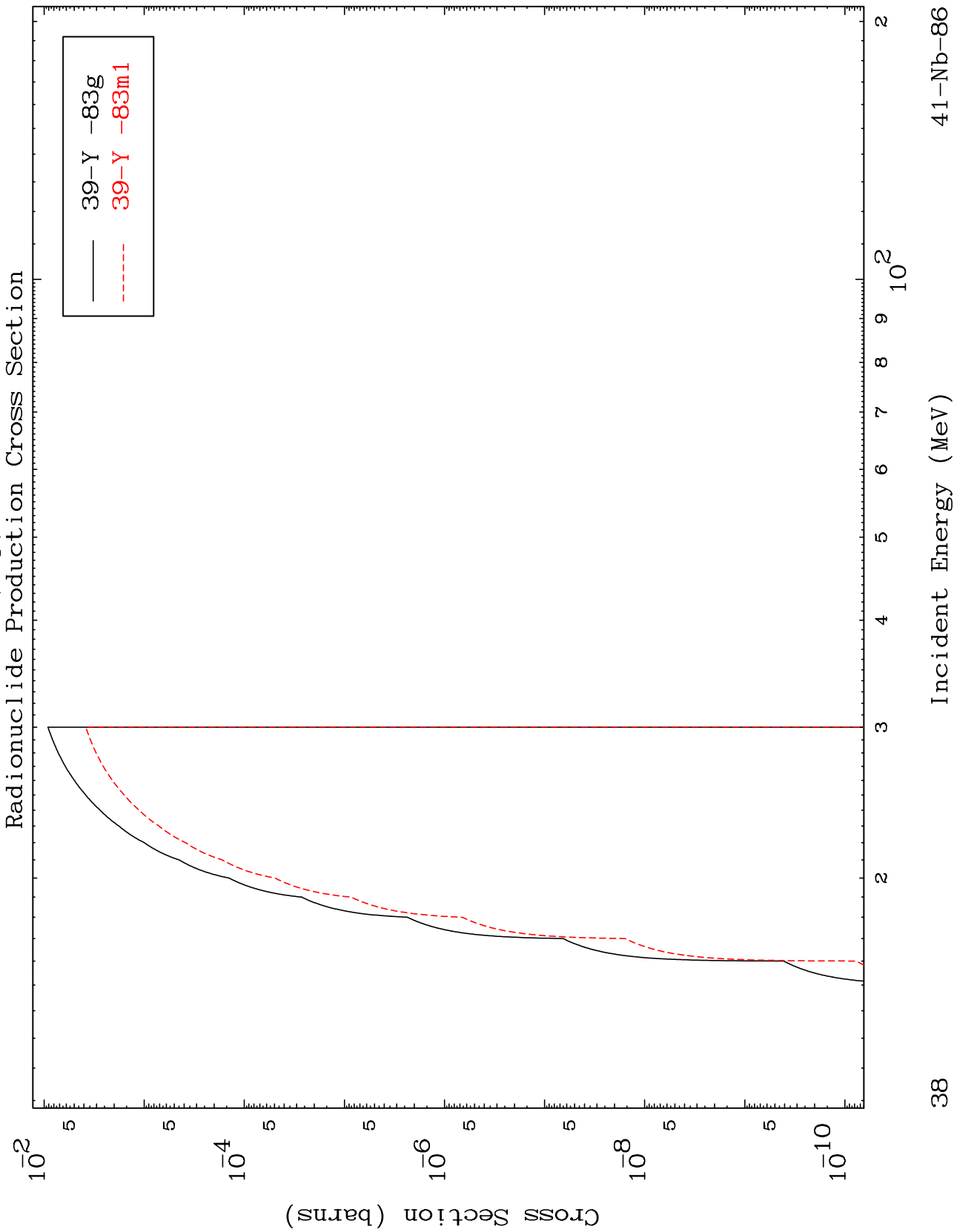
Incident Energy (MeV)

41-Nb-86

MAT 4104

(n,p) t

41-Nb-86



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Incident Energy (MeV)

41-Nb-86