

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

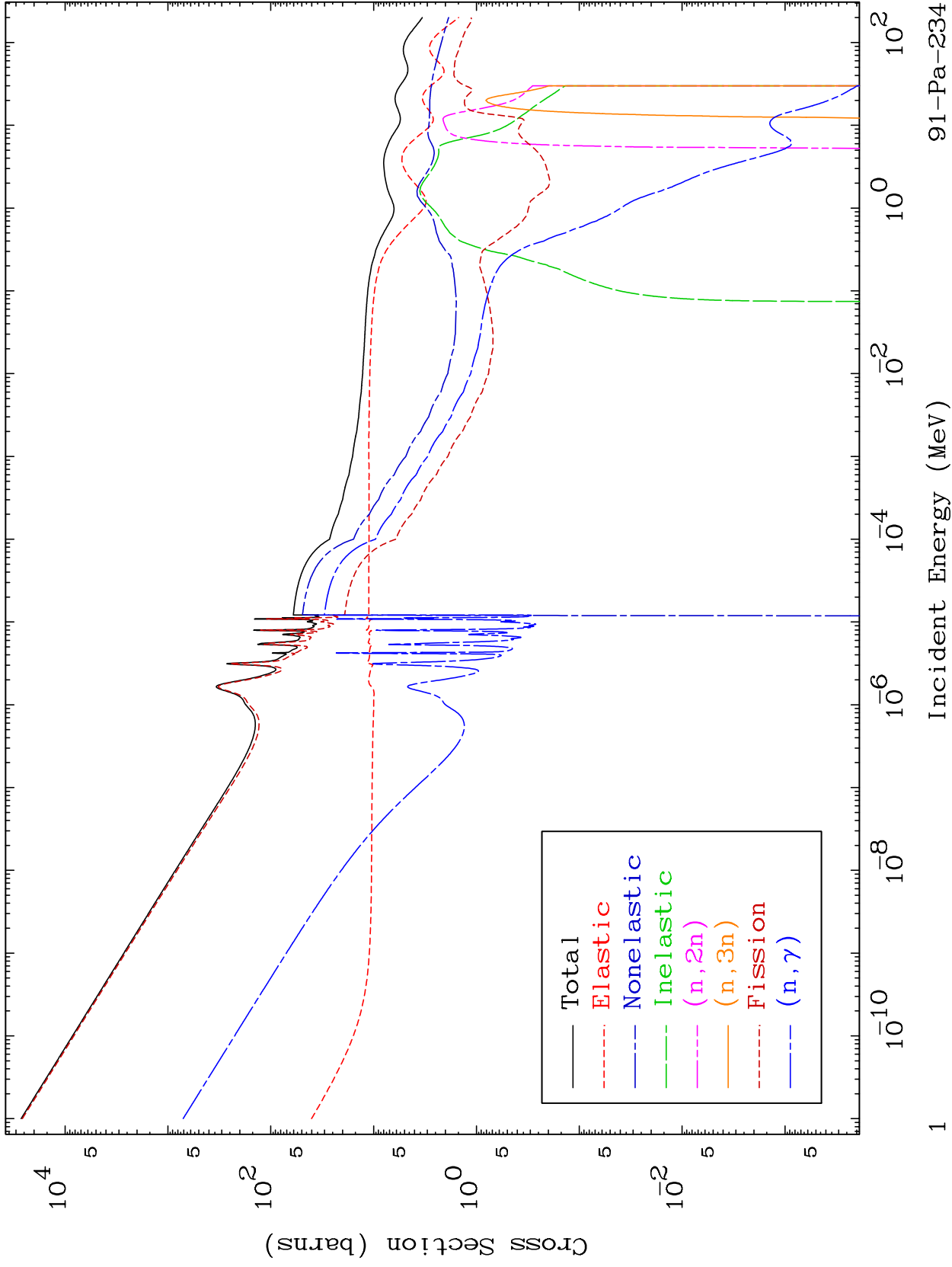
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Press Mouse Button to Start

MAT 9140

Neutron Major
293 Kelvin Cross Sections

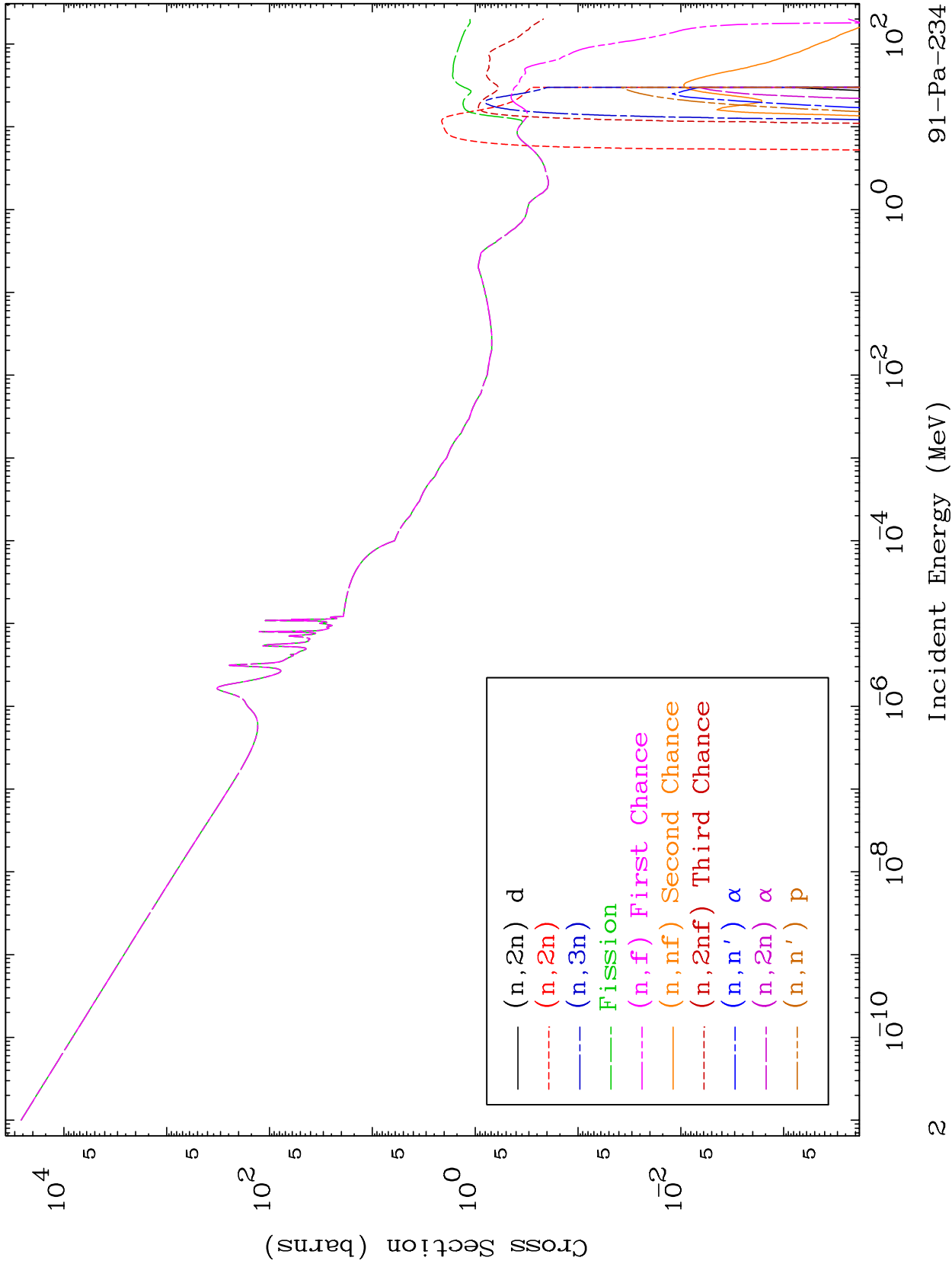
91-Pa-234



MAT 9140

Neutron Absorption
293 Kelvin Cross Sections

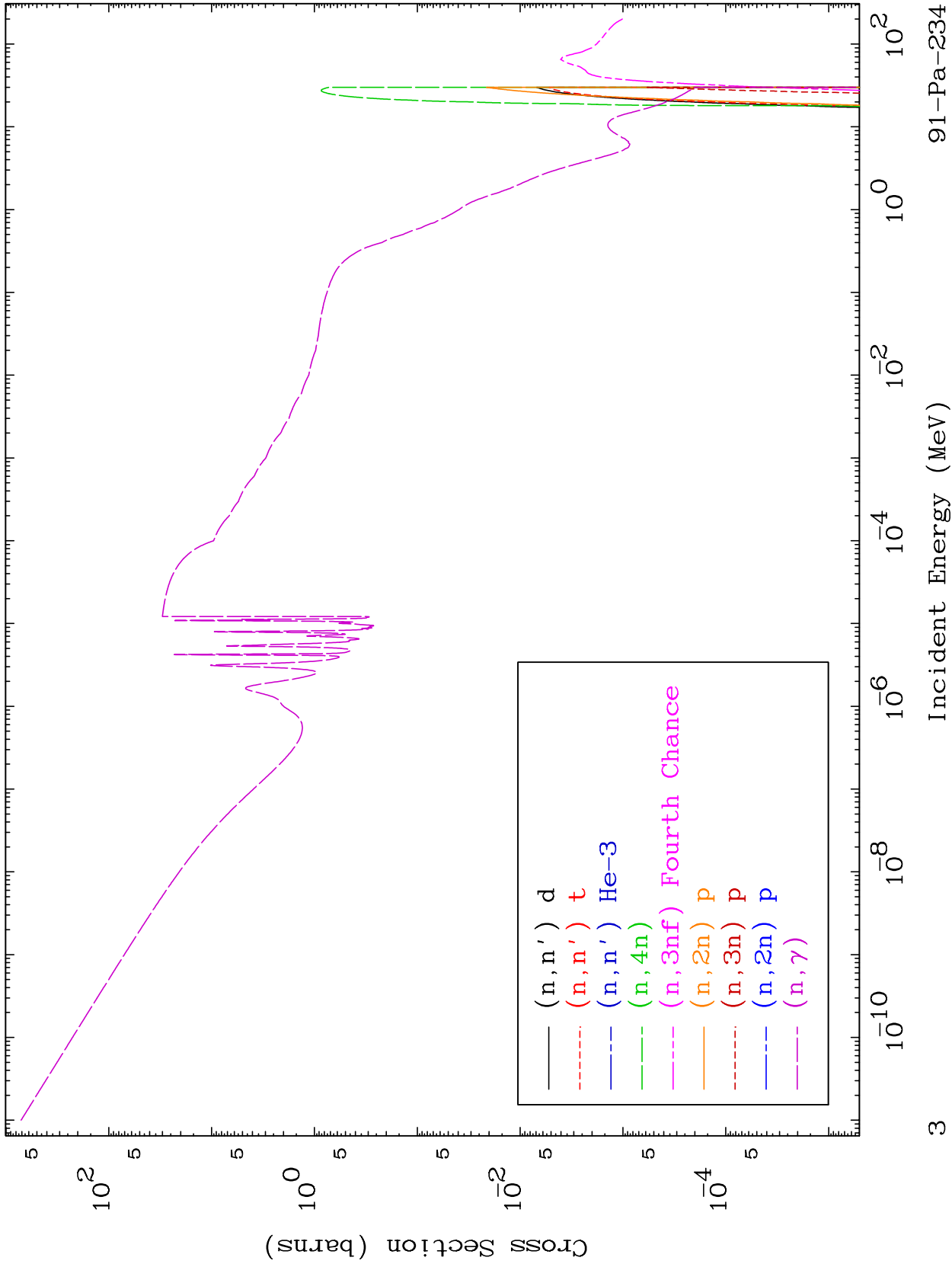
91-Pa-234



MAT 9140

Neutron Absorption
293 Kelvin Cross Sections

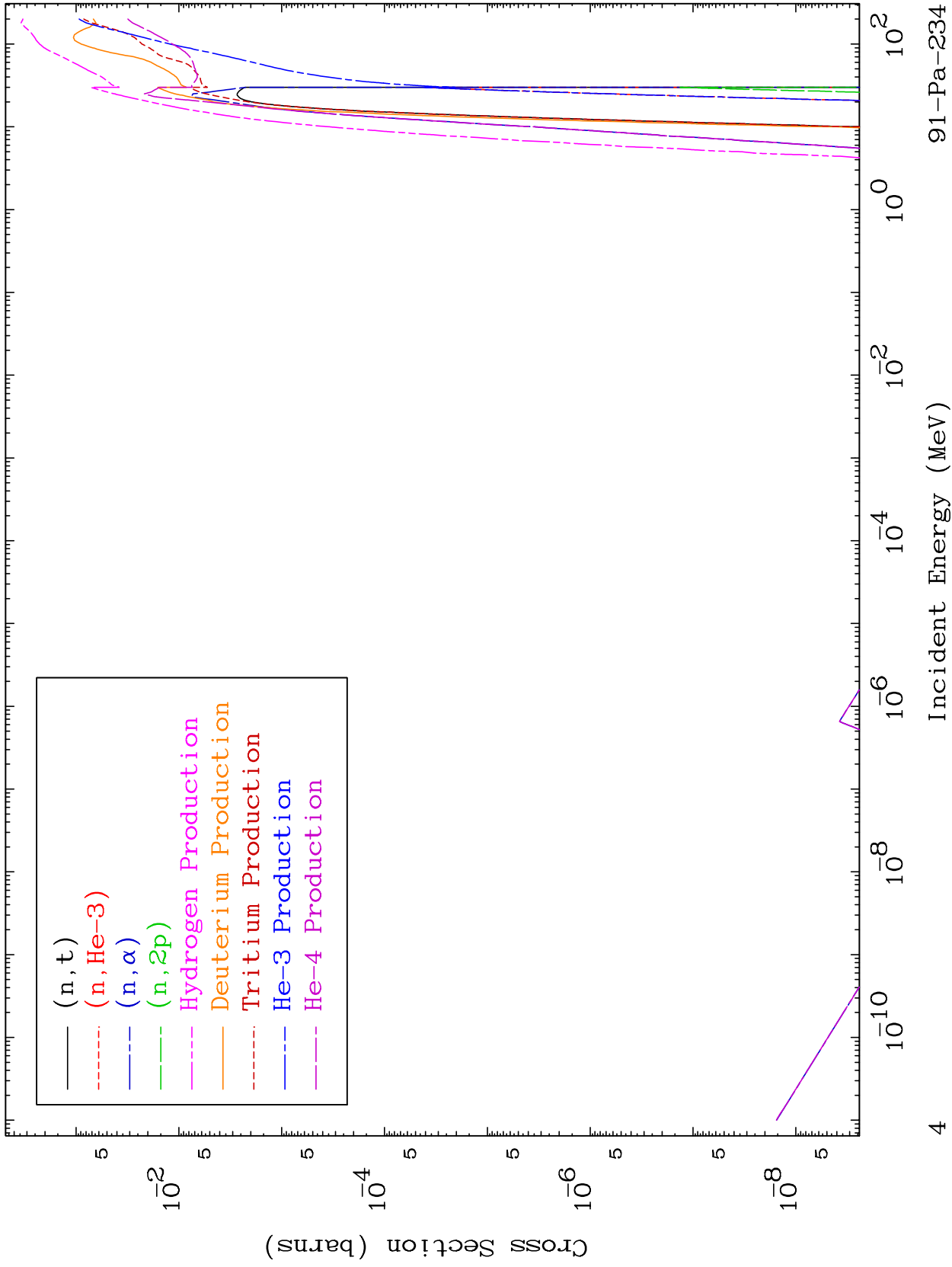
91-Pa-234



MAT 9140

Neutron Absorption
293 Kelvin Cross Sections

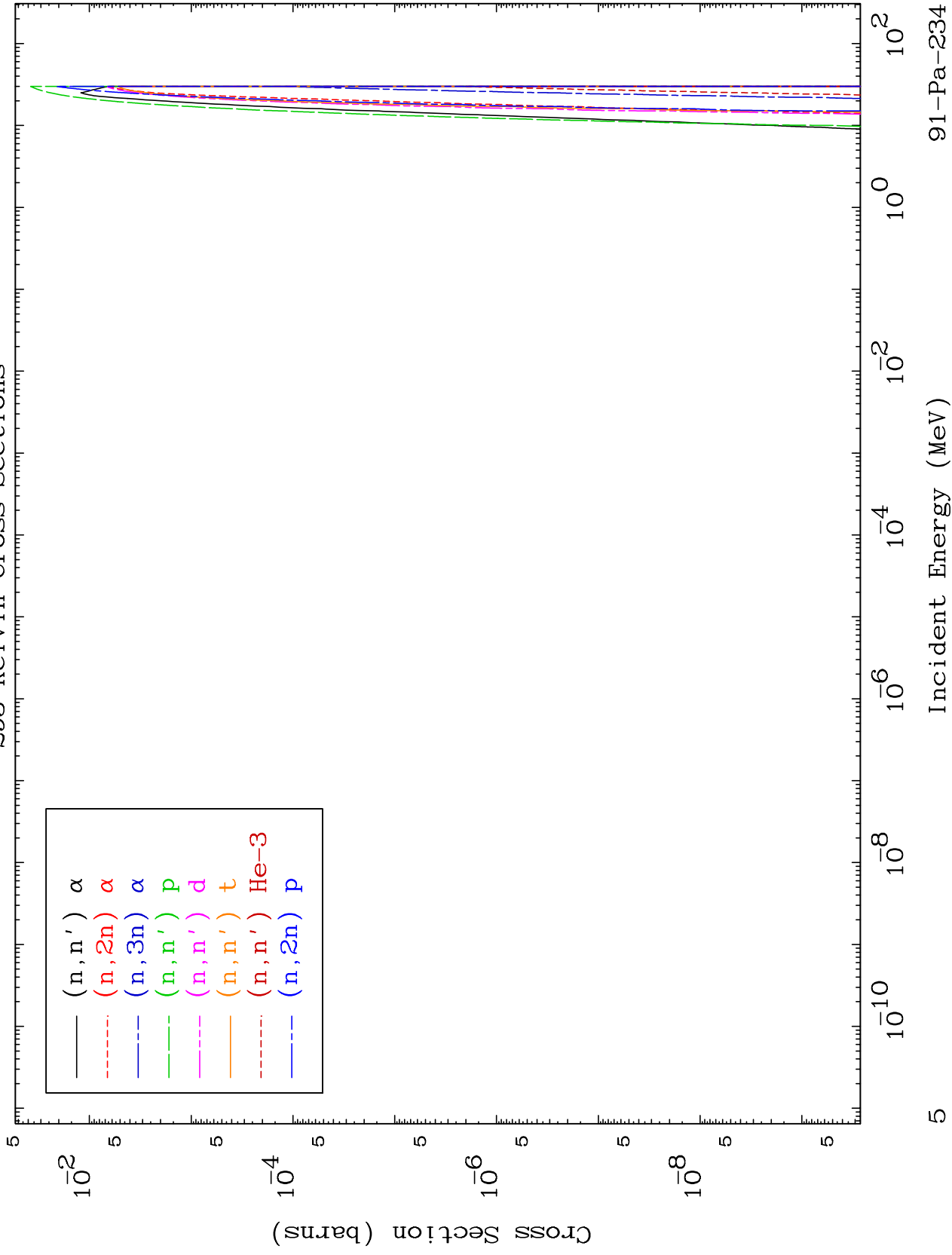
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MAT 9140

Charged Particle
293 Kelvin Cross Sections

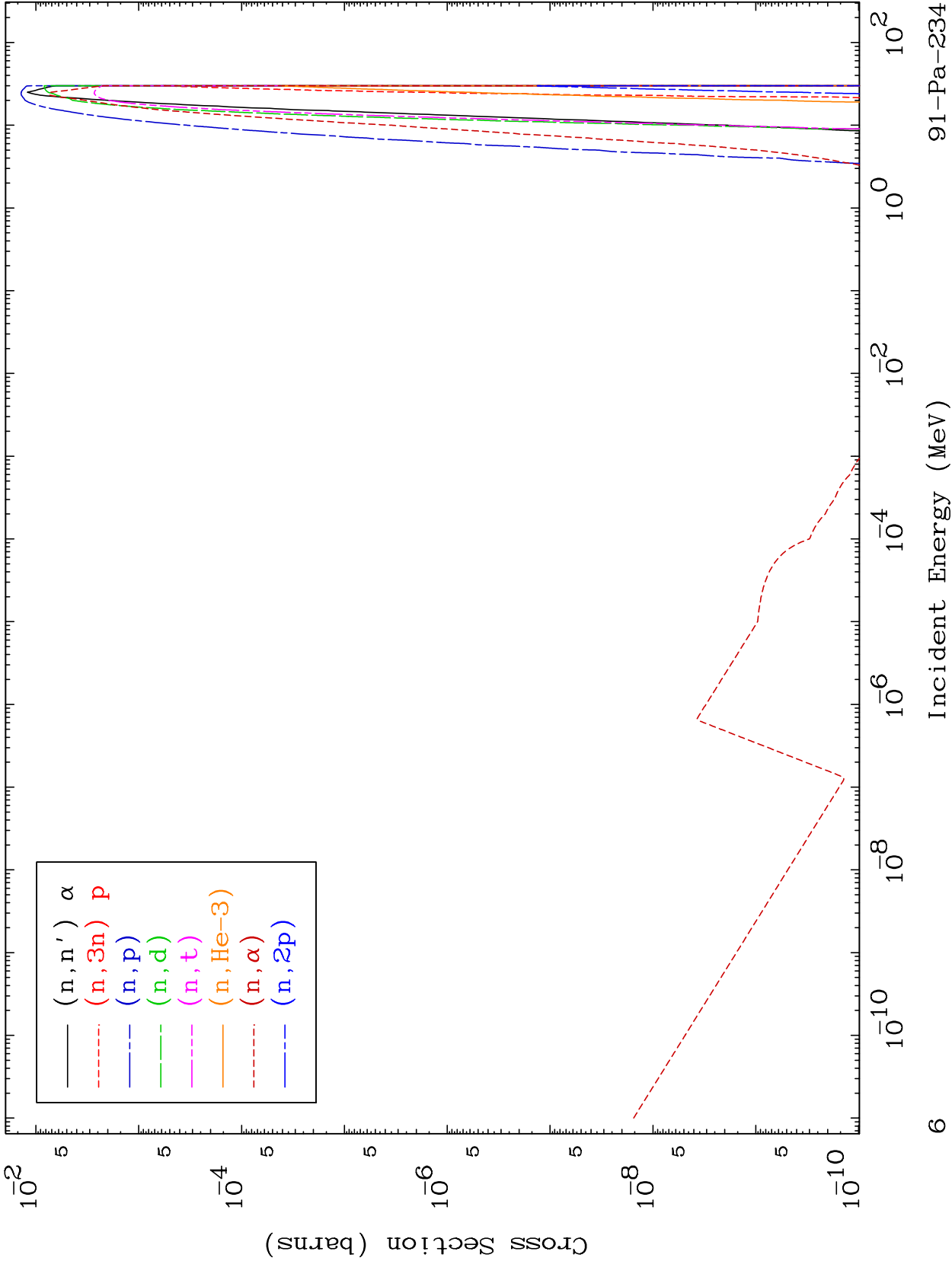
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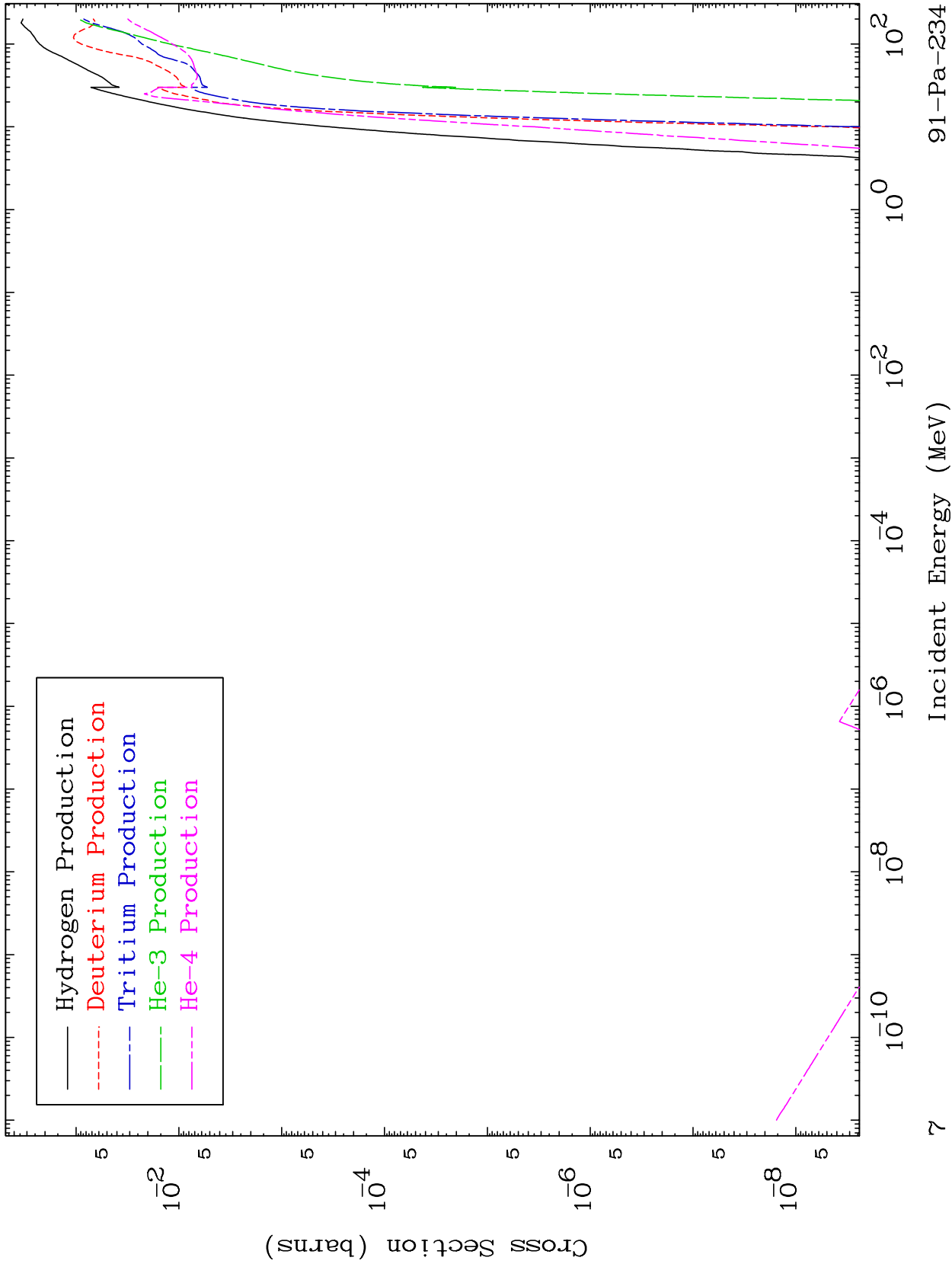


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Charged Particle
293 Kelvin Cross Sections

91-Pa-234

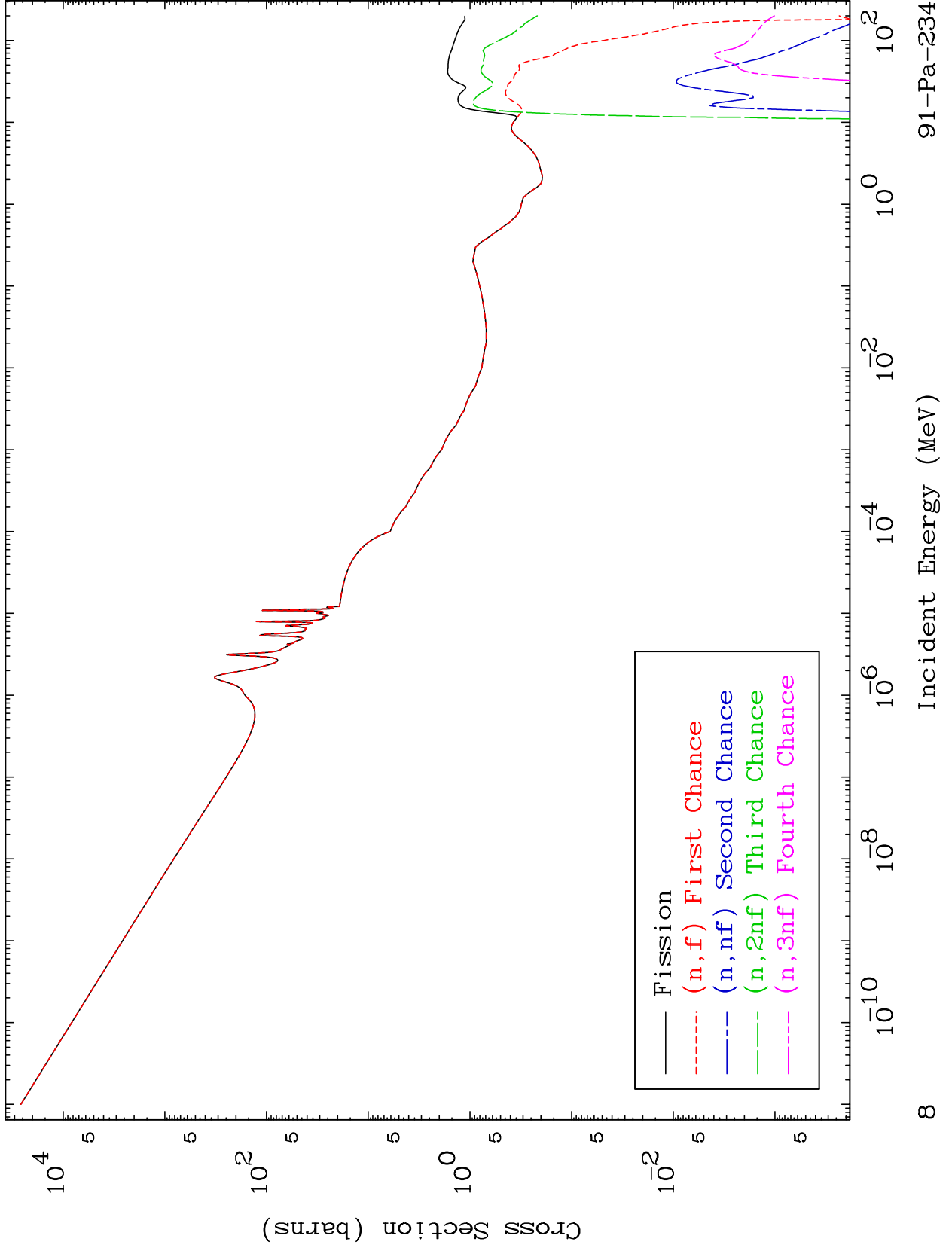




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Fission
293 Kelvin Cross Sections

91-Pa-234

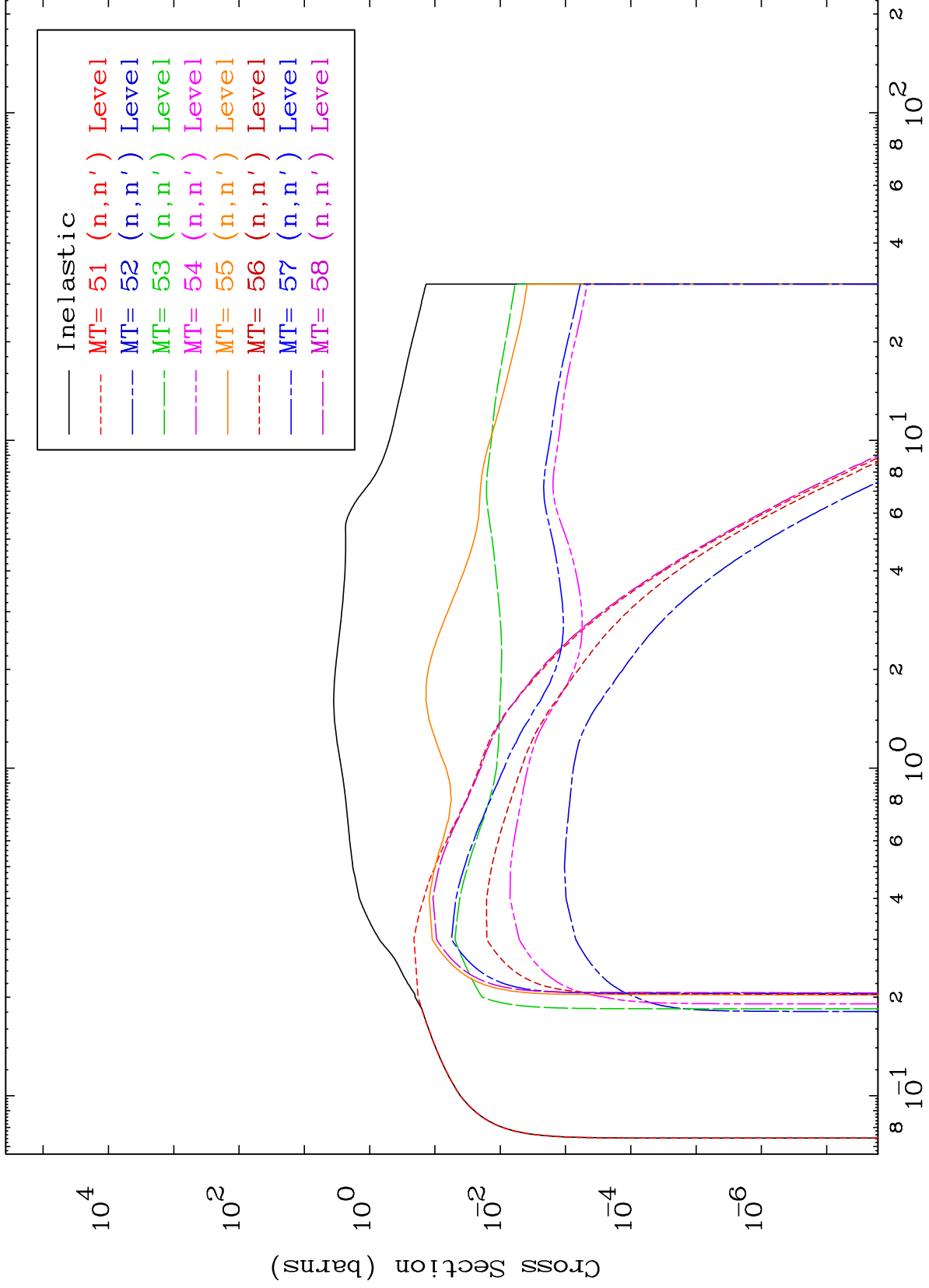


MAT 9140

(n,n') Levels

91-Pa-234

293 Kelvin Cross Sections



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Incident Energy (MeV)

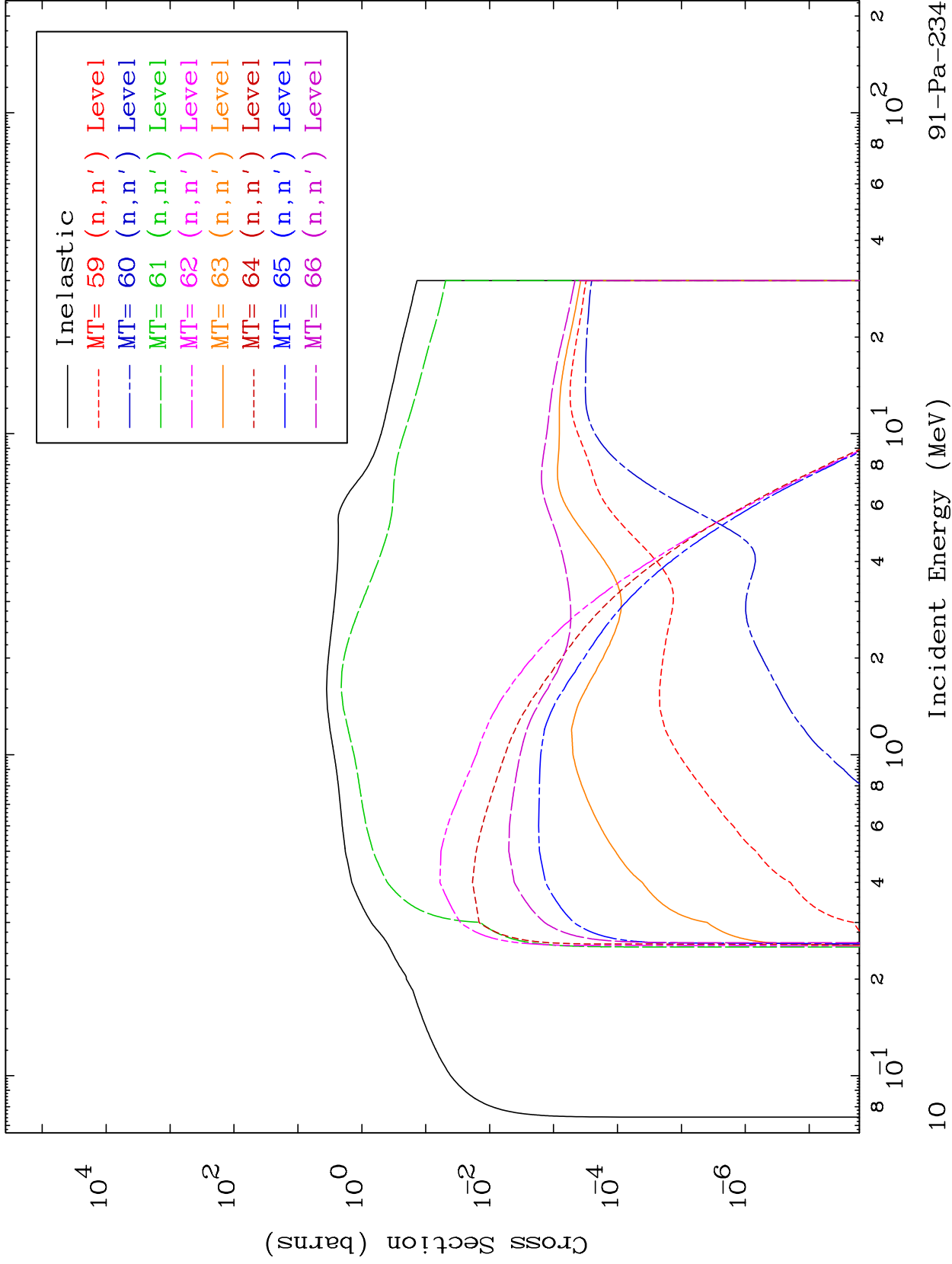
91-Pa-234

MAT 9140

(n,n') Levels

91-Pa-234

293 Kelvin Cross Sections

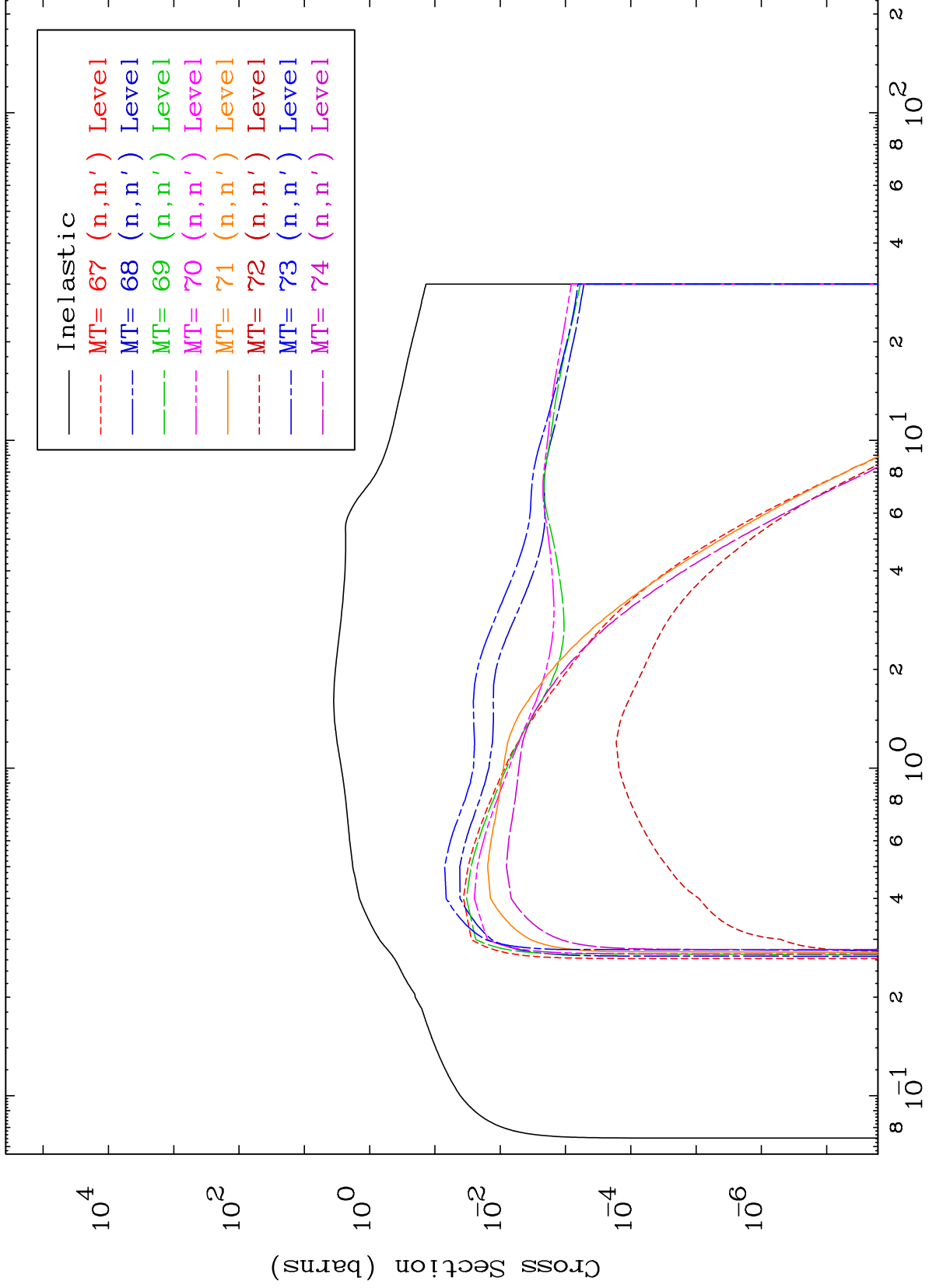


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(n,n') Levels

91-Pa-234

293 Kelvin Cross Sections



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Incident Energy (MeV)

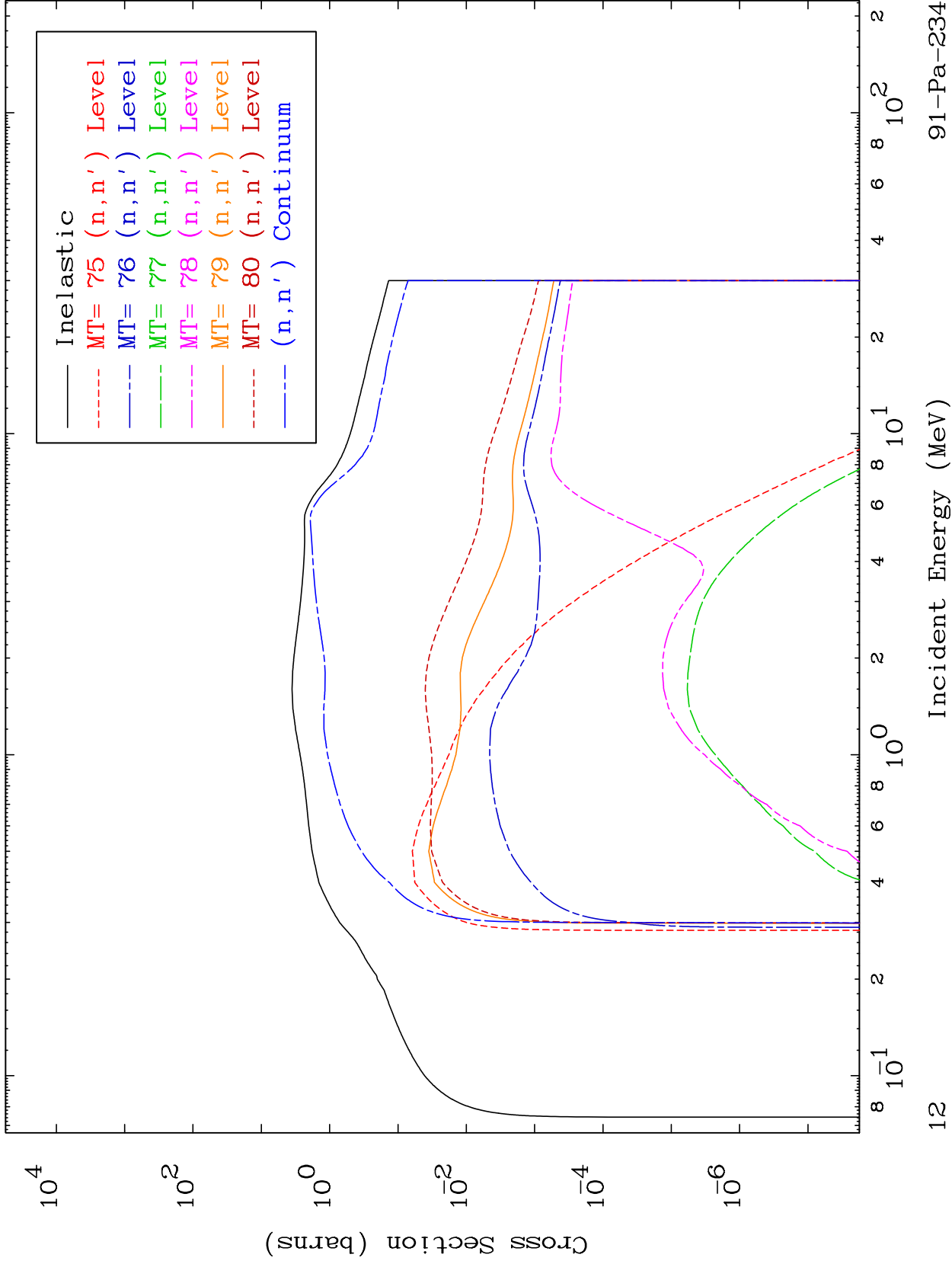
91-Pa-234

MAT 9140

(n,n') Levels

293 Kelvin Cross Sections

91-Pa-234

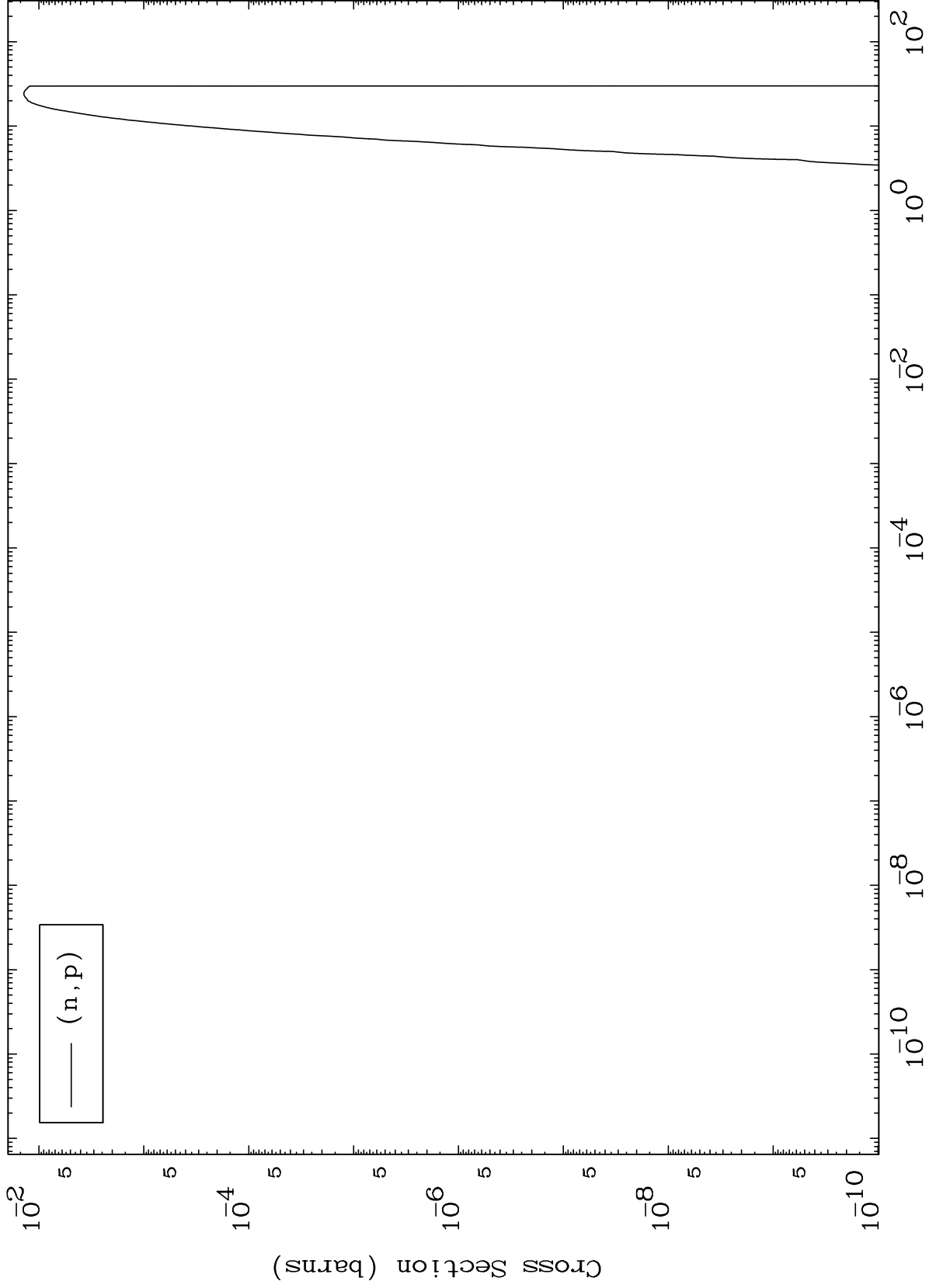


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(n,p) Levels
293 Kelvin Cross Sections

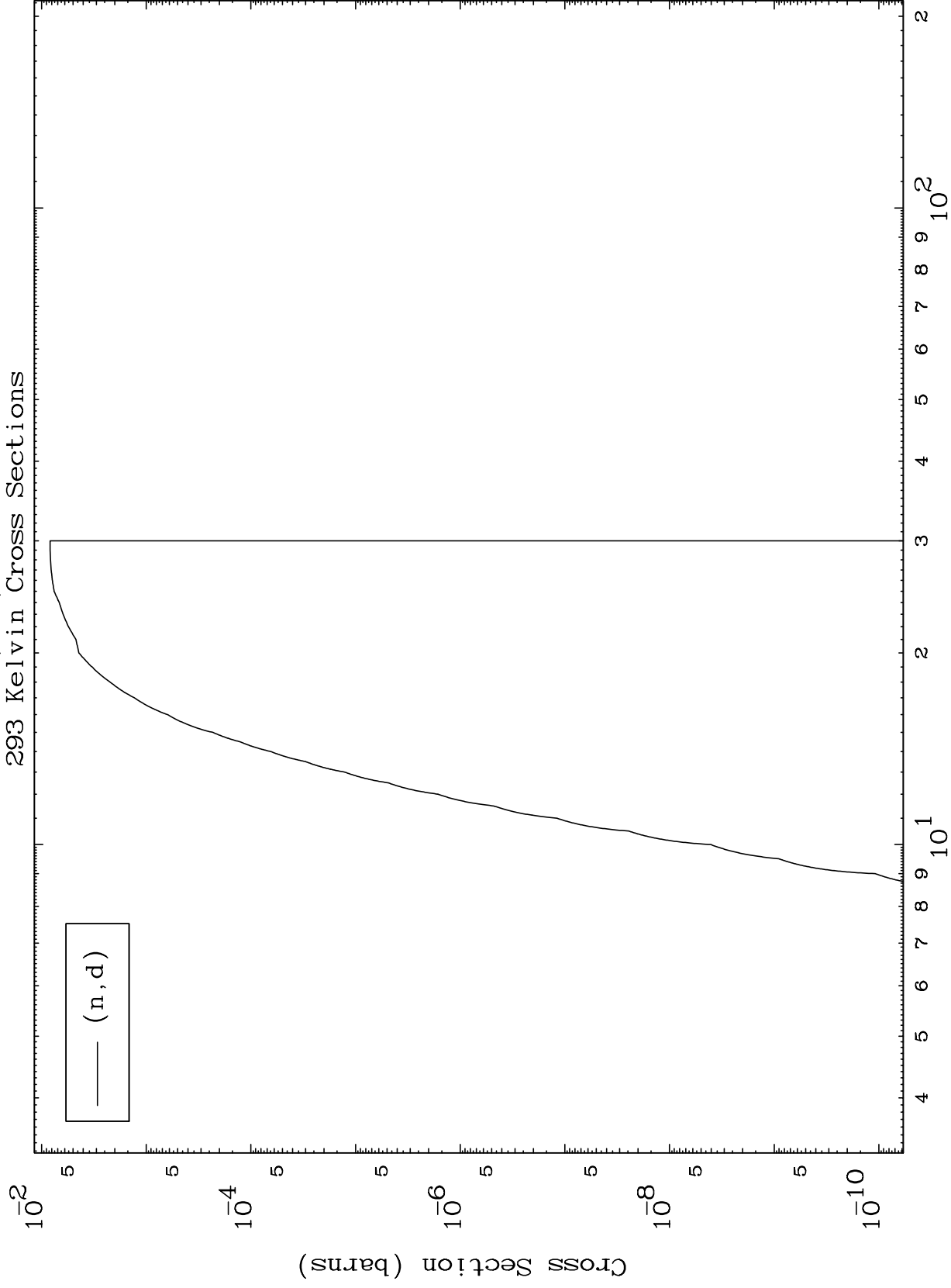
91-Pa-234



MAT 9140

(n,d) Levels
293 Kelvin Cross Sections

91-Pa-234



14

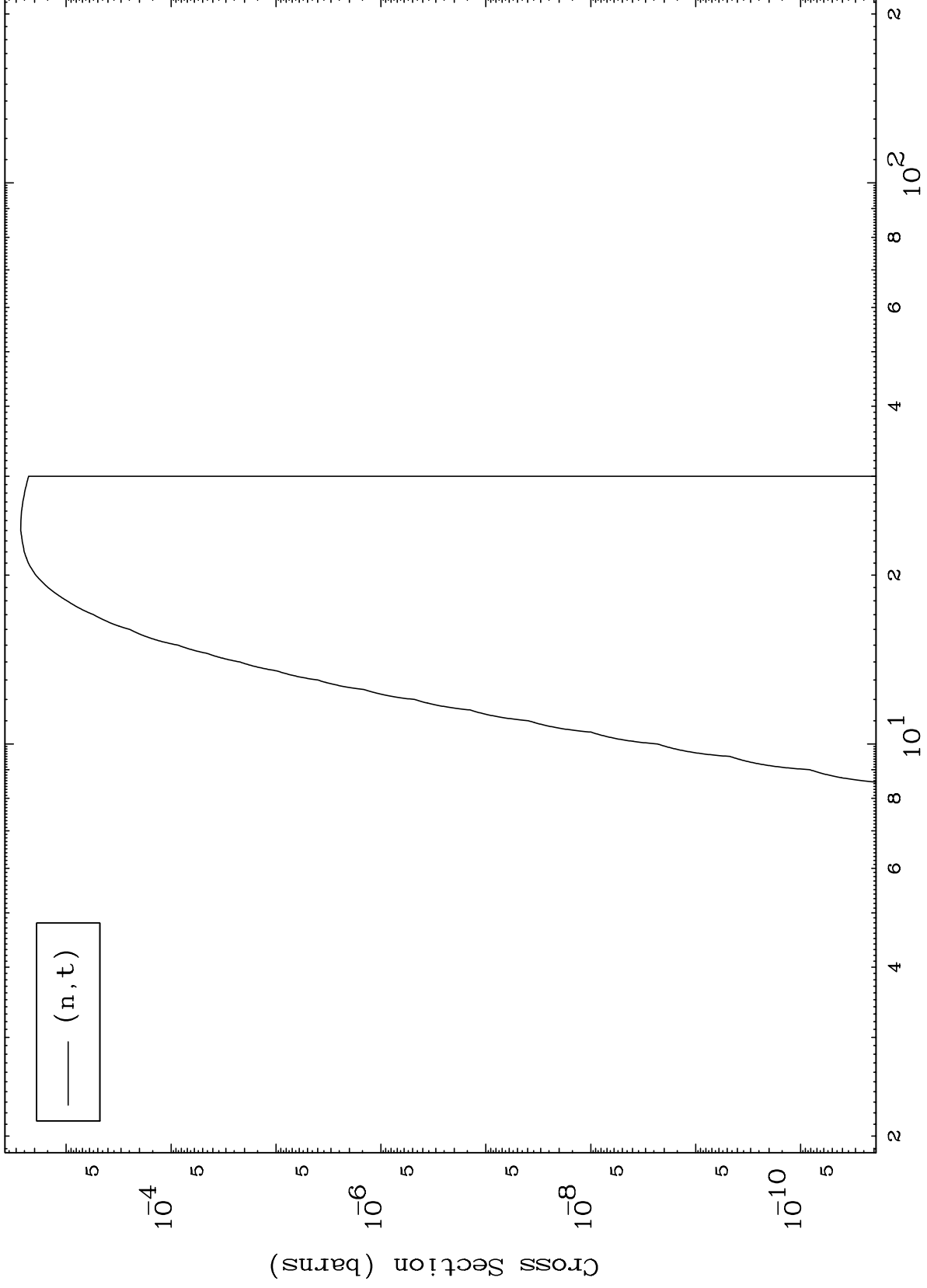
Incident Energy (MeV)

91-Pa-234

MAT 9140

(n,t) Levels
293 Kelvin Cross Sections

91-Pa-234



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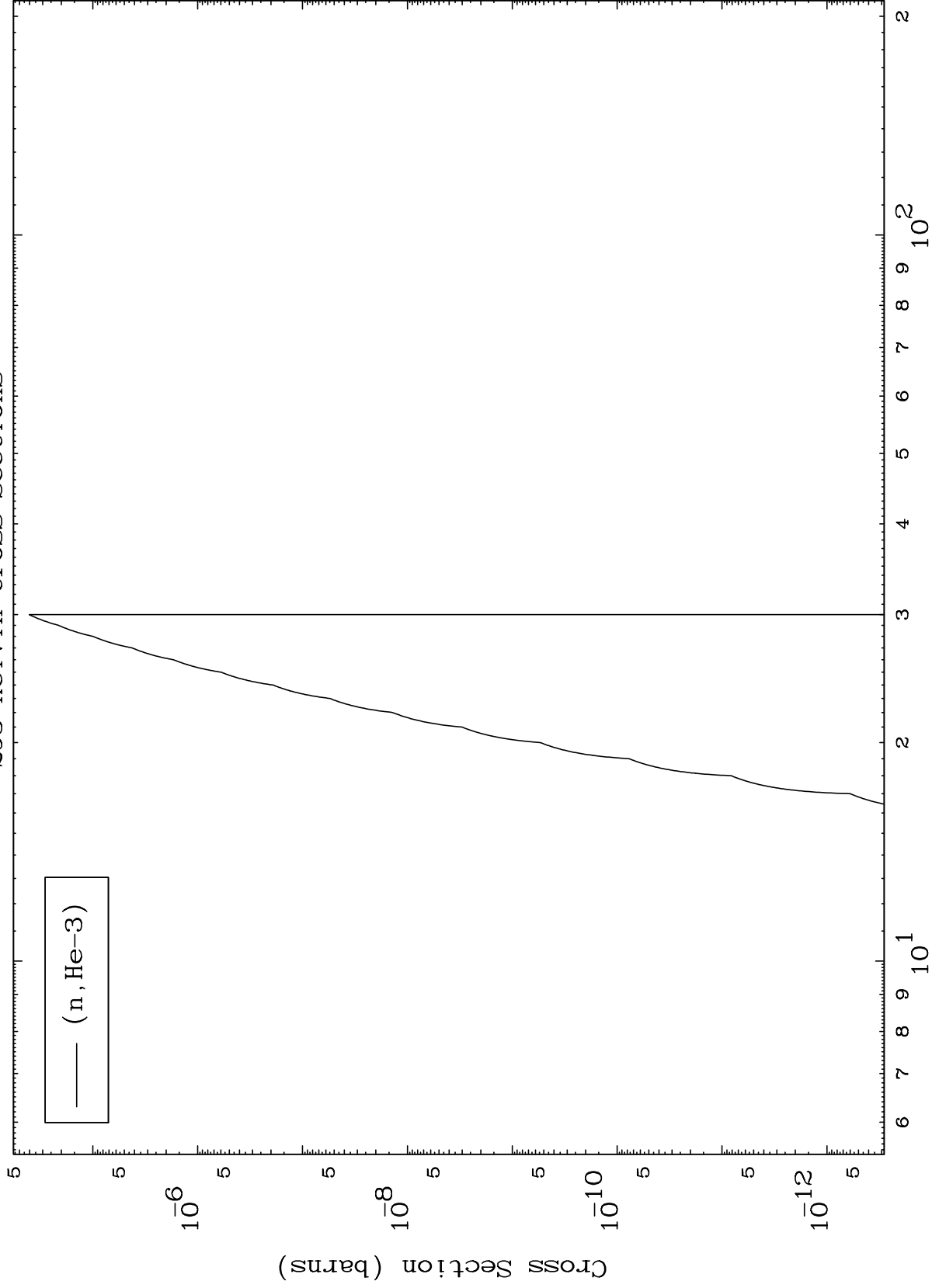
Incident Energy (MeV)

91-Pa-234

MAT 9140

(n,He3) Levels
293 Kelvin Cross Sections

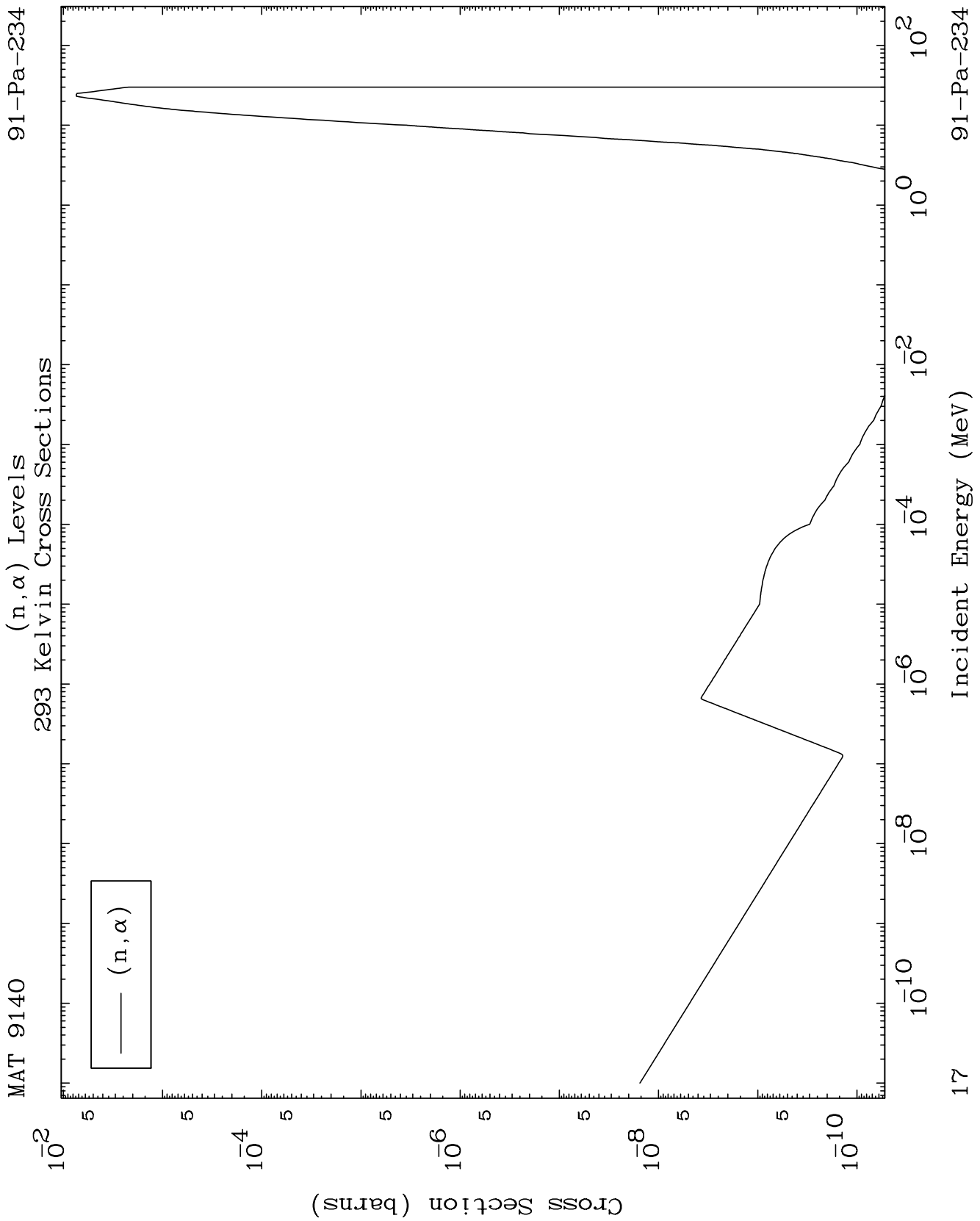
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Incident Energy (MeV)

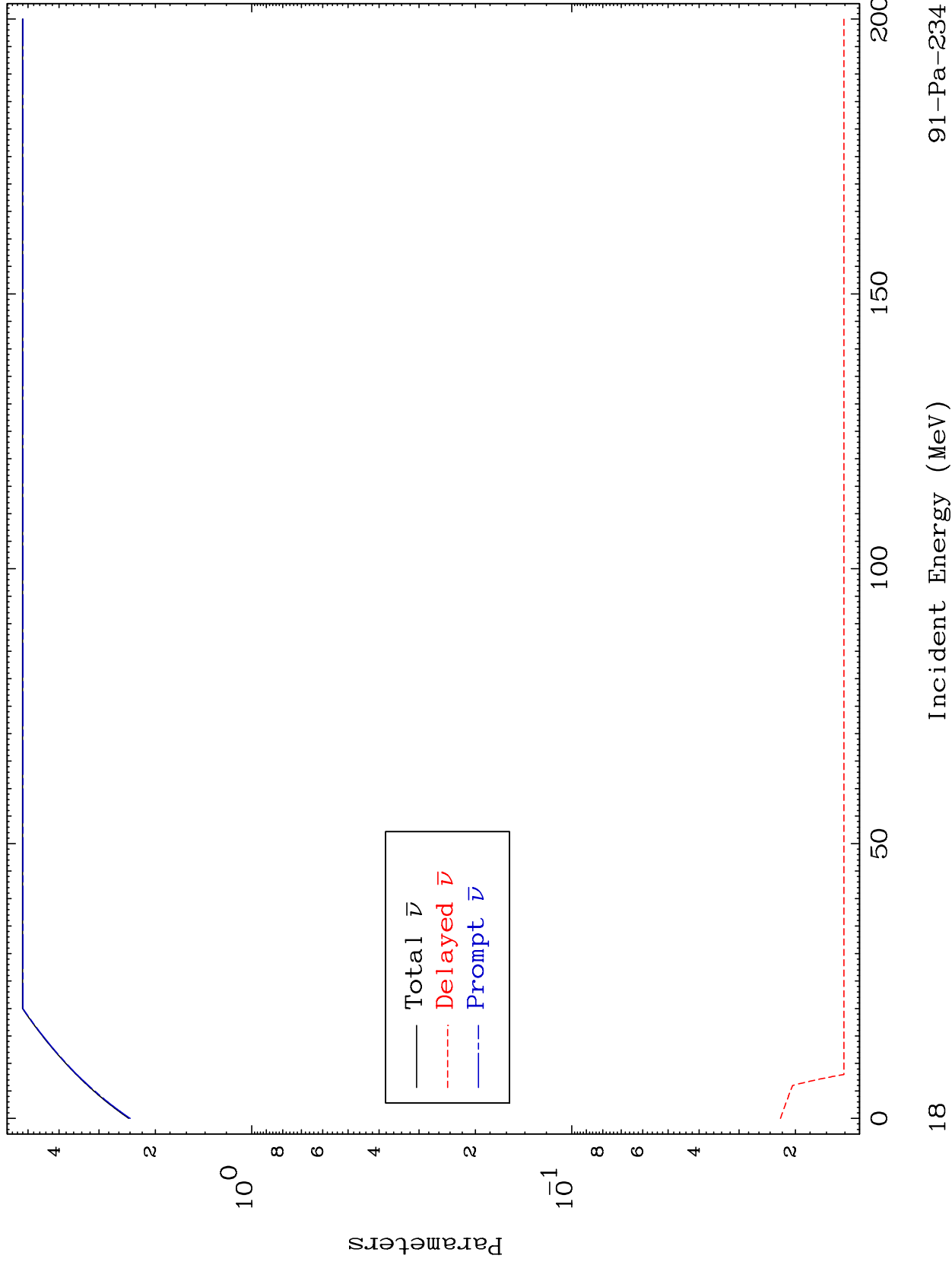
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Energy Release
Parameters

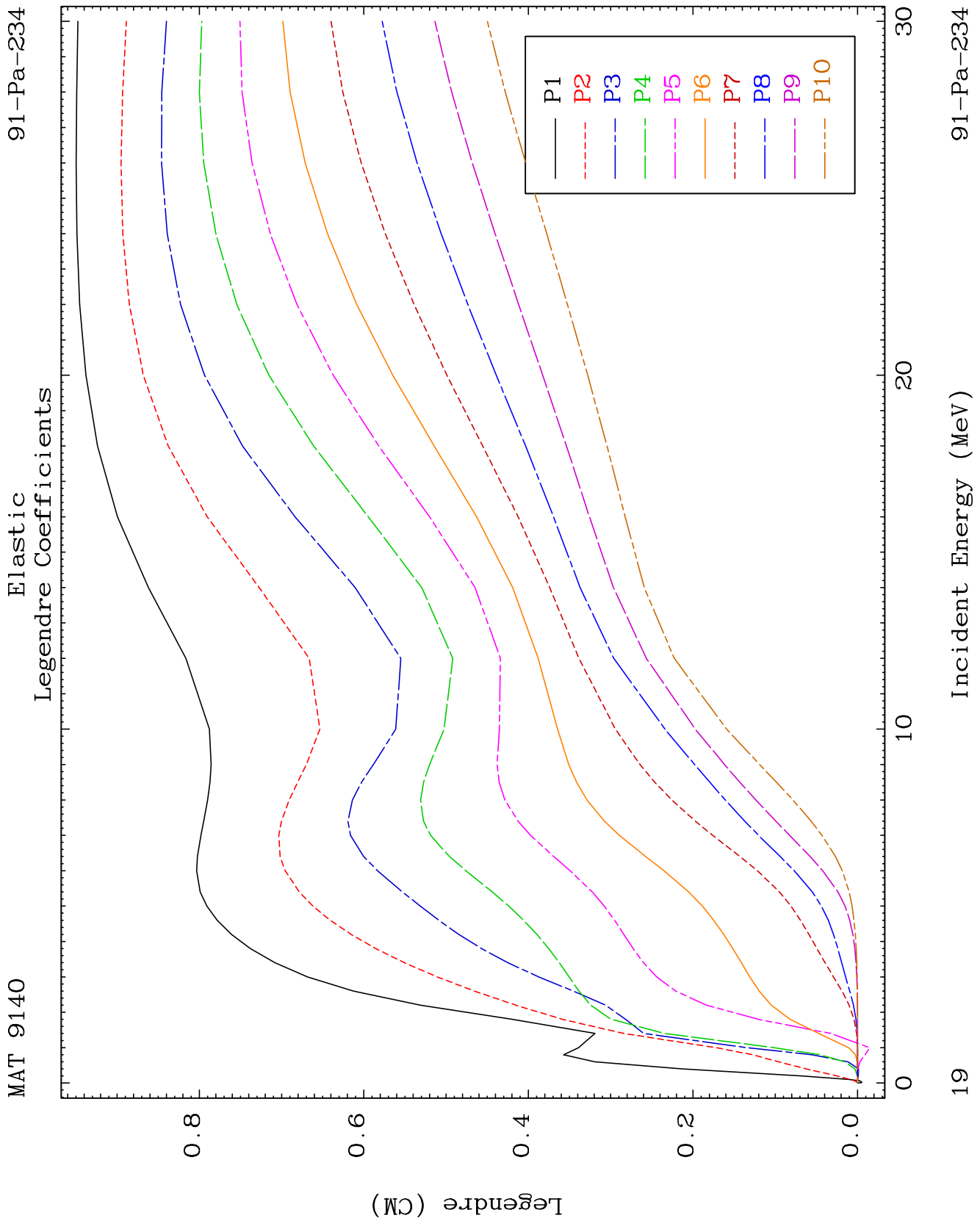
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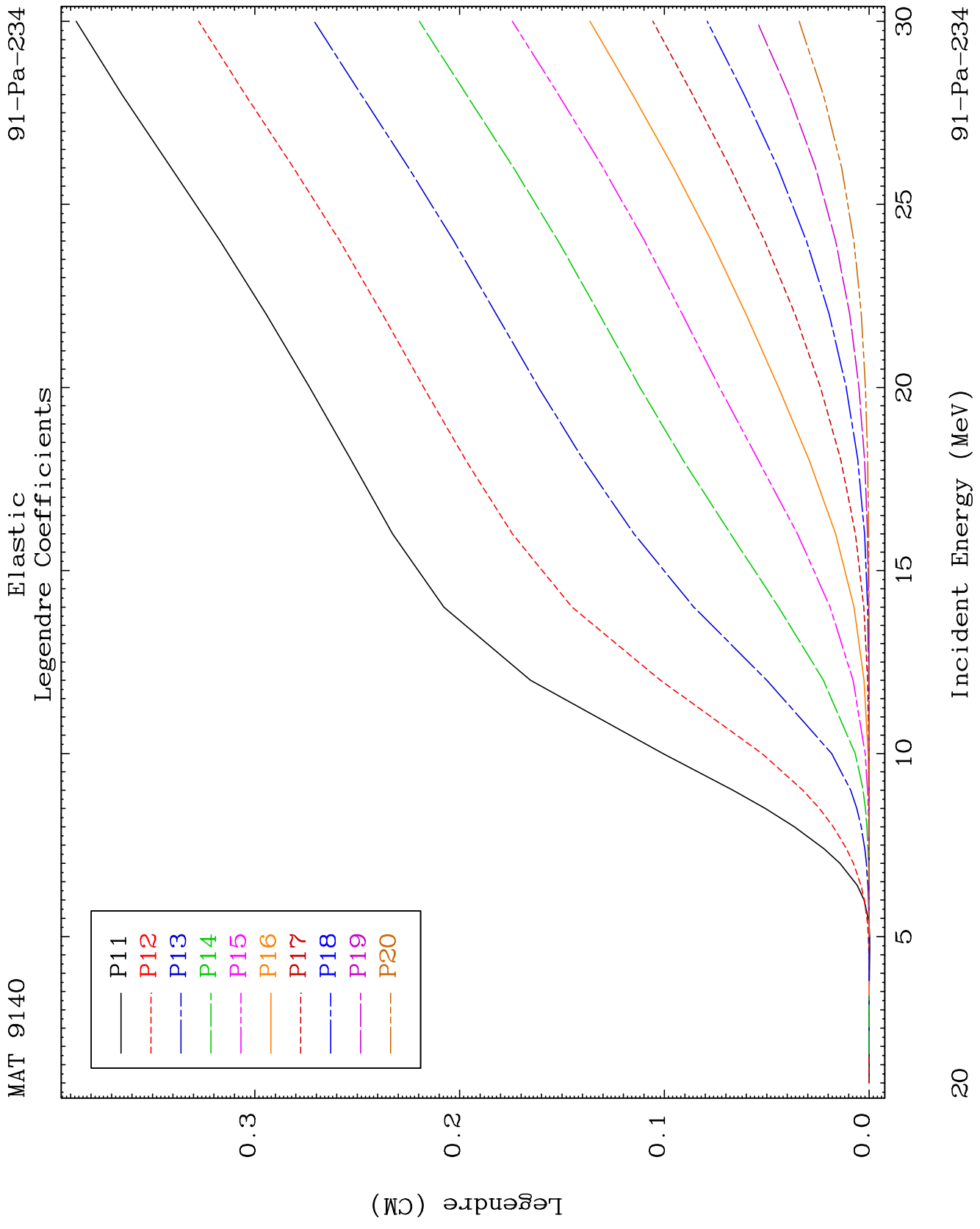


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Incident Energy (MeV)

91-Pa-234

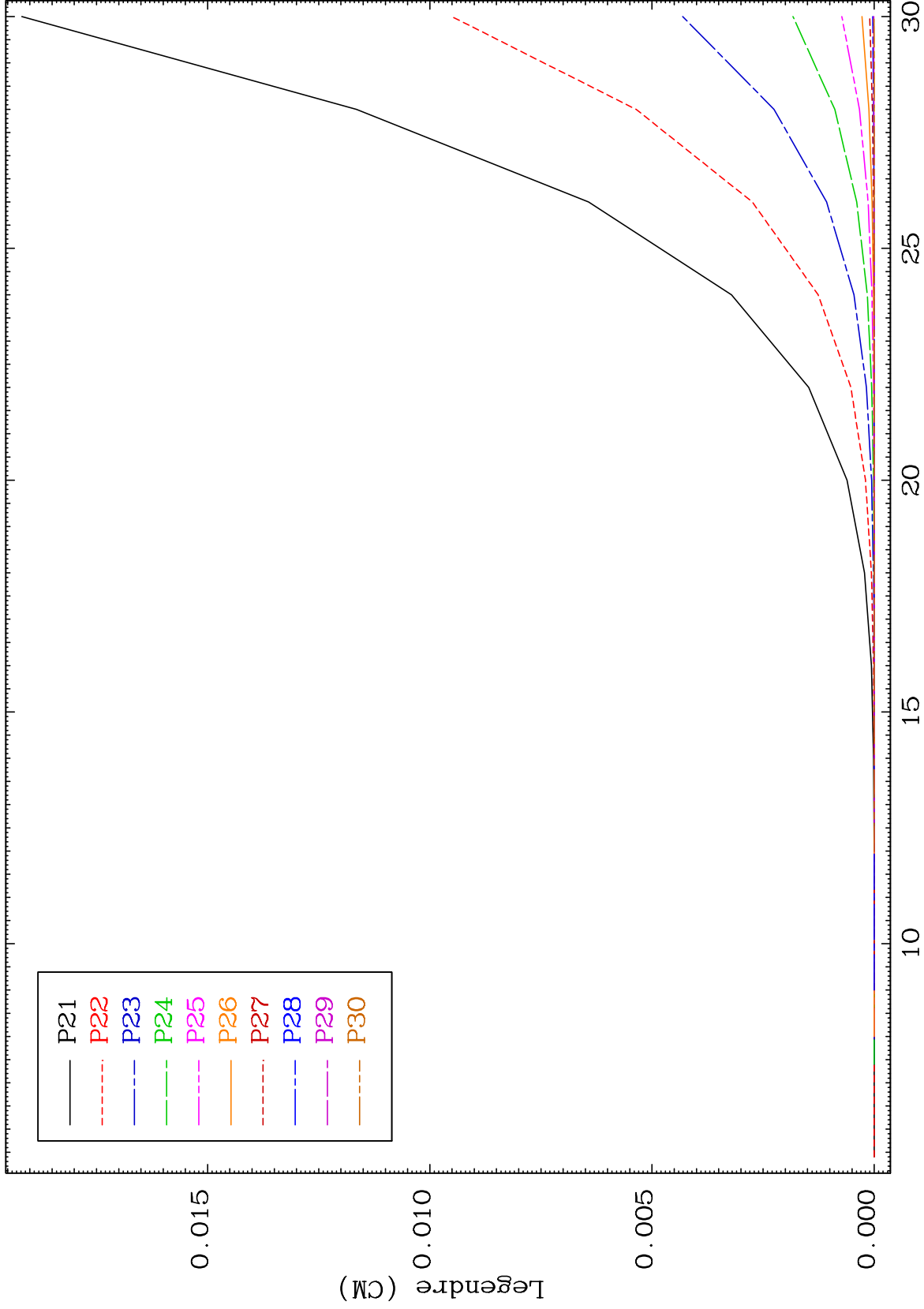




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Elastic Legendre Coefficients

91-Pa-234



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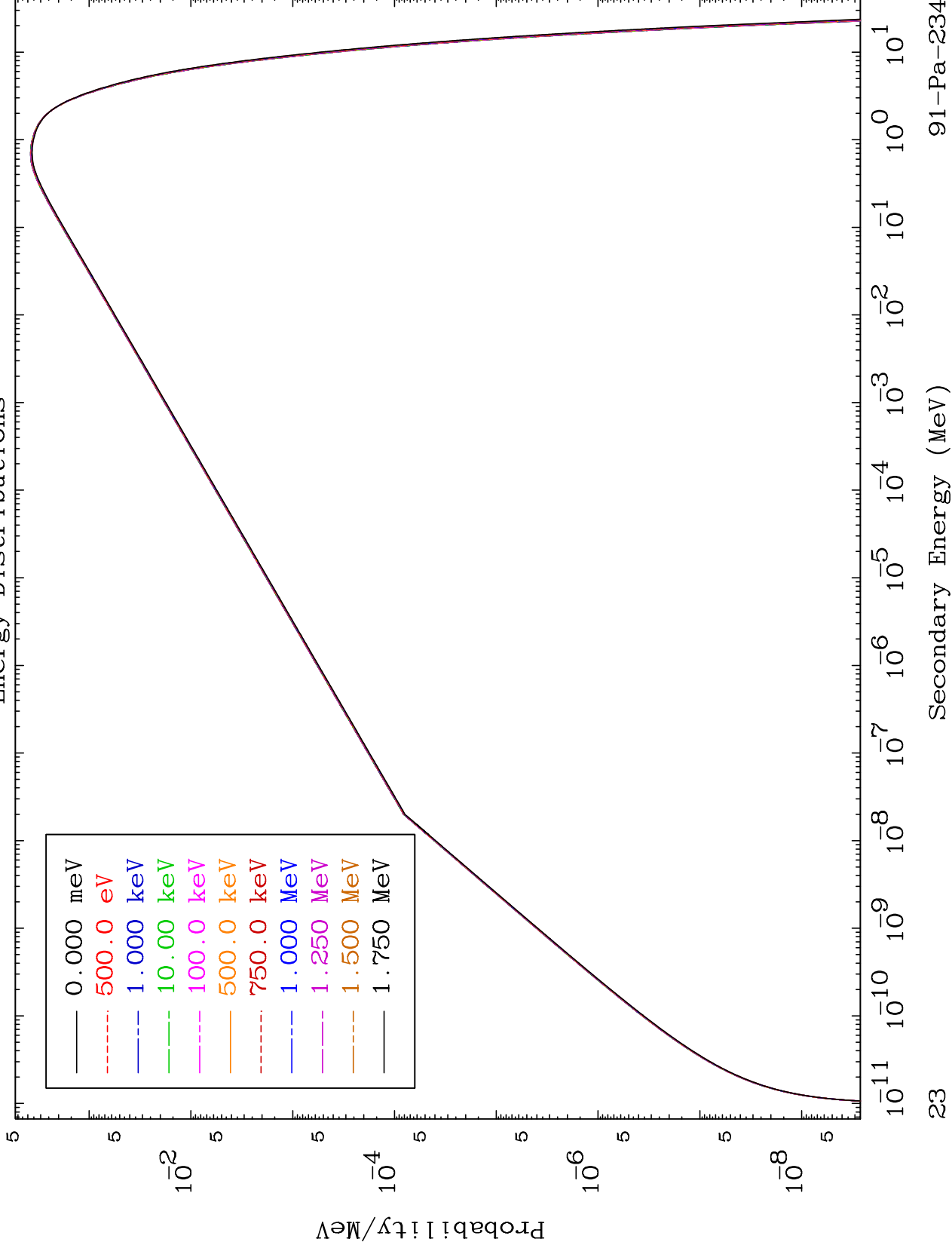
Incident Energy (MeV)

91-Pa-234

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Fission Energy Distributions

91-Pa-234



91-Pa-234

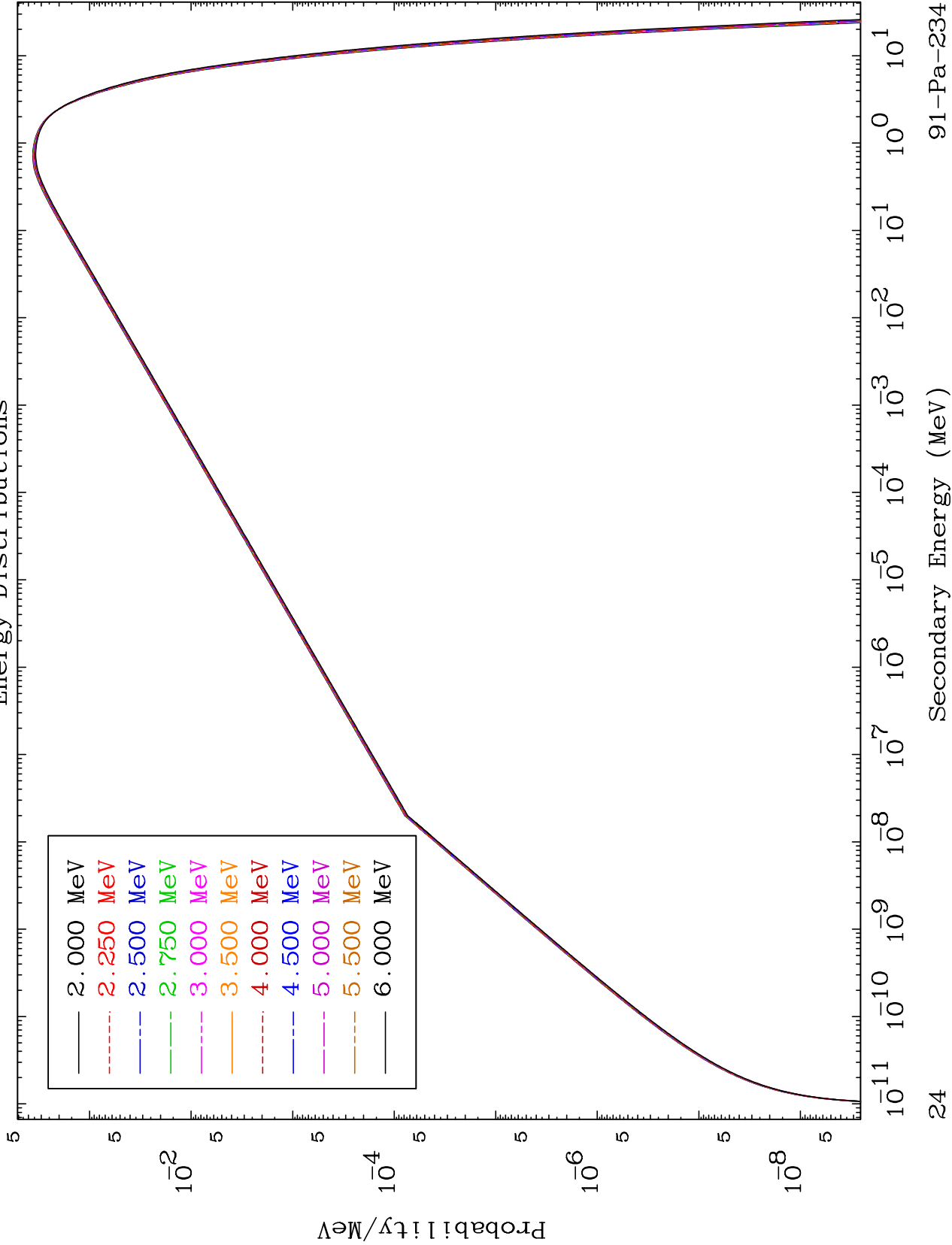
Secondary Energy (MeV)

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Fission Energy Distributions

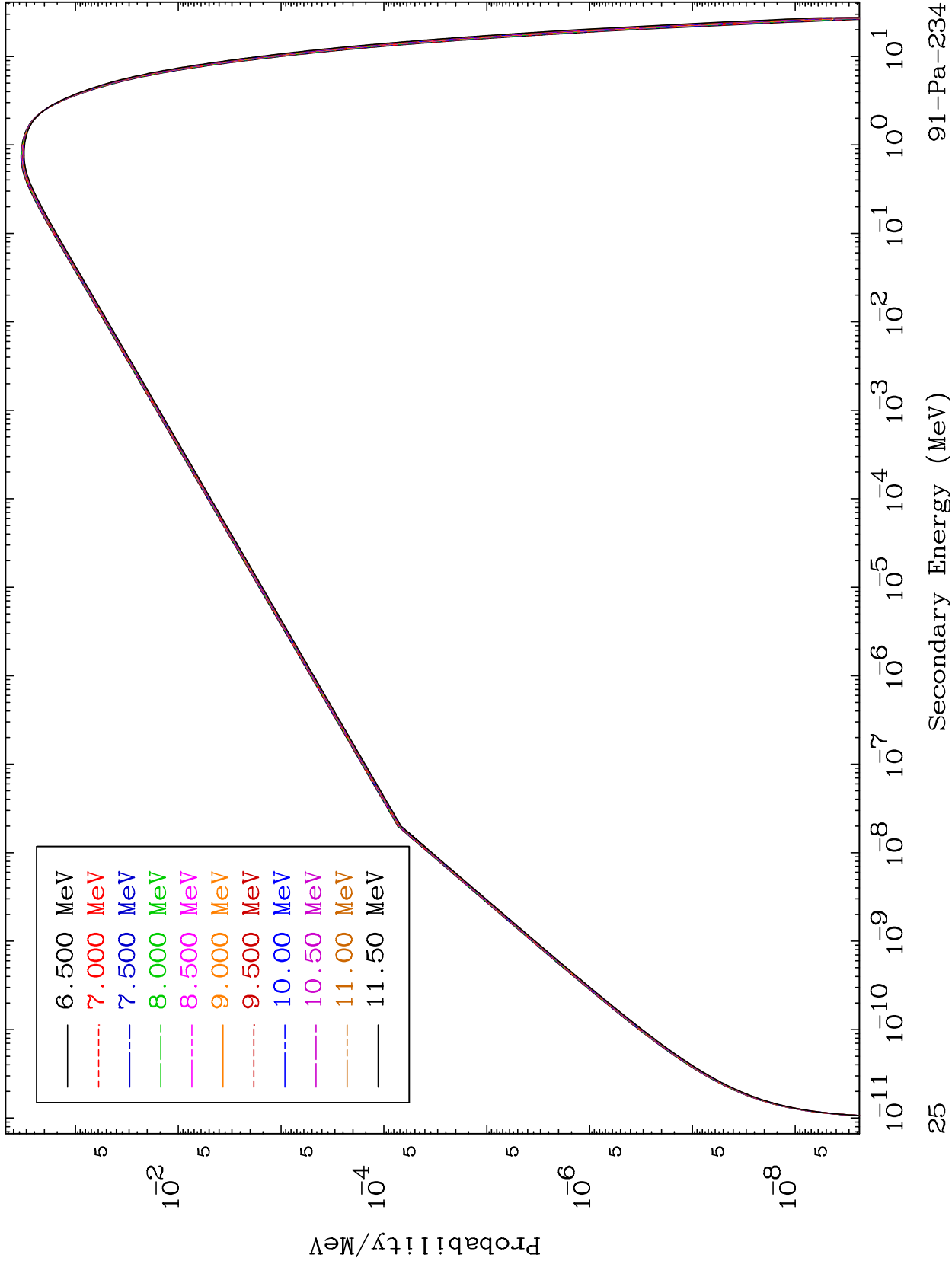
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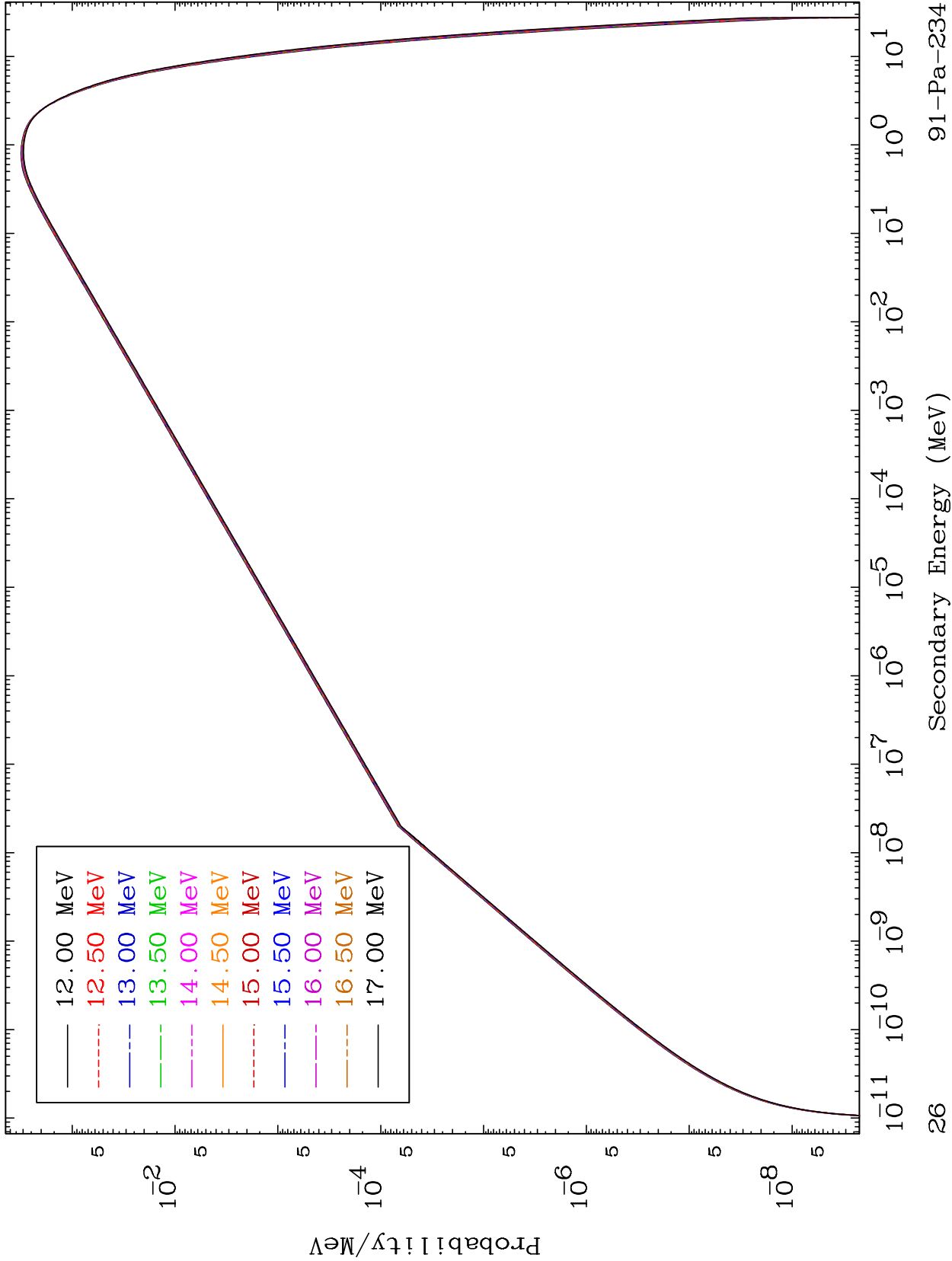
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Fission Energy Distributions

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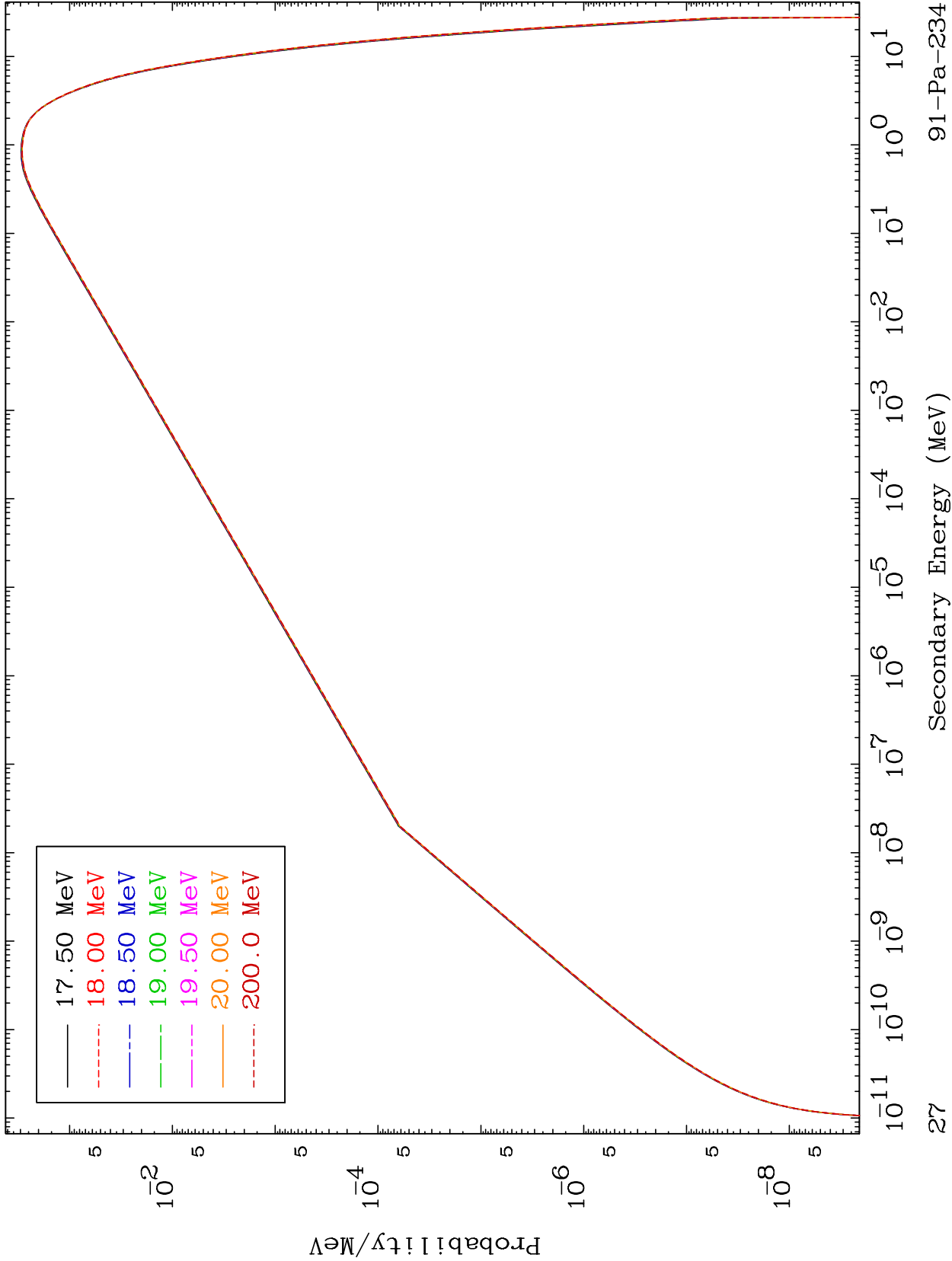
Fission Energy Distributions



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Fission Energy Distributions

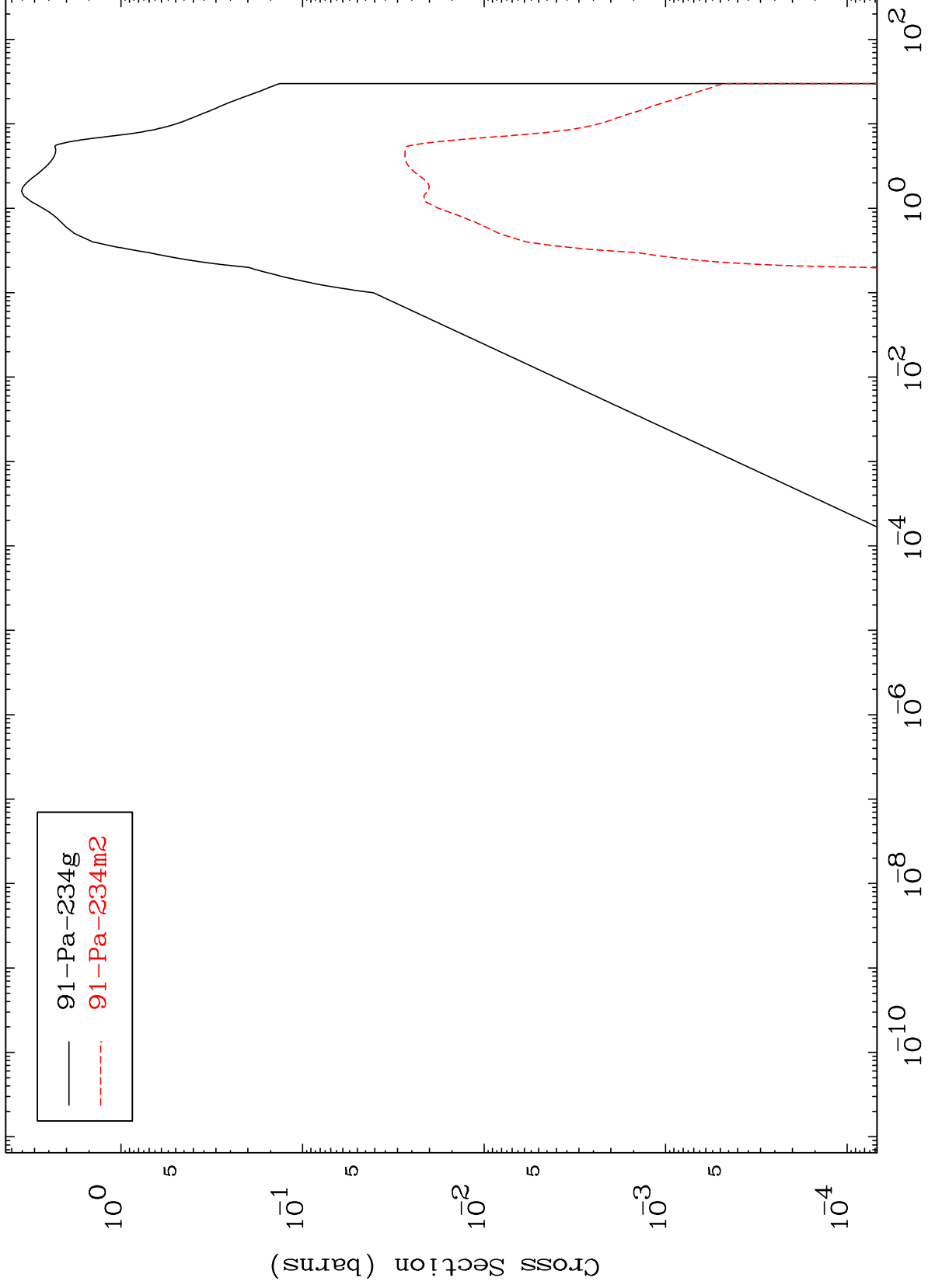
91-Pa-234



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Inelastic
Radionuclide Production Cross Section

91-Pa-234



91-Pa-234