

Program EVALPLOT
(Version 2021-1)

by

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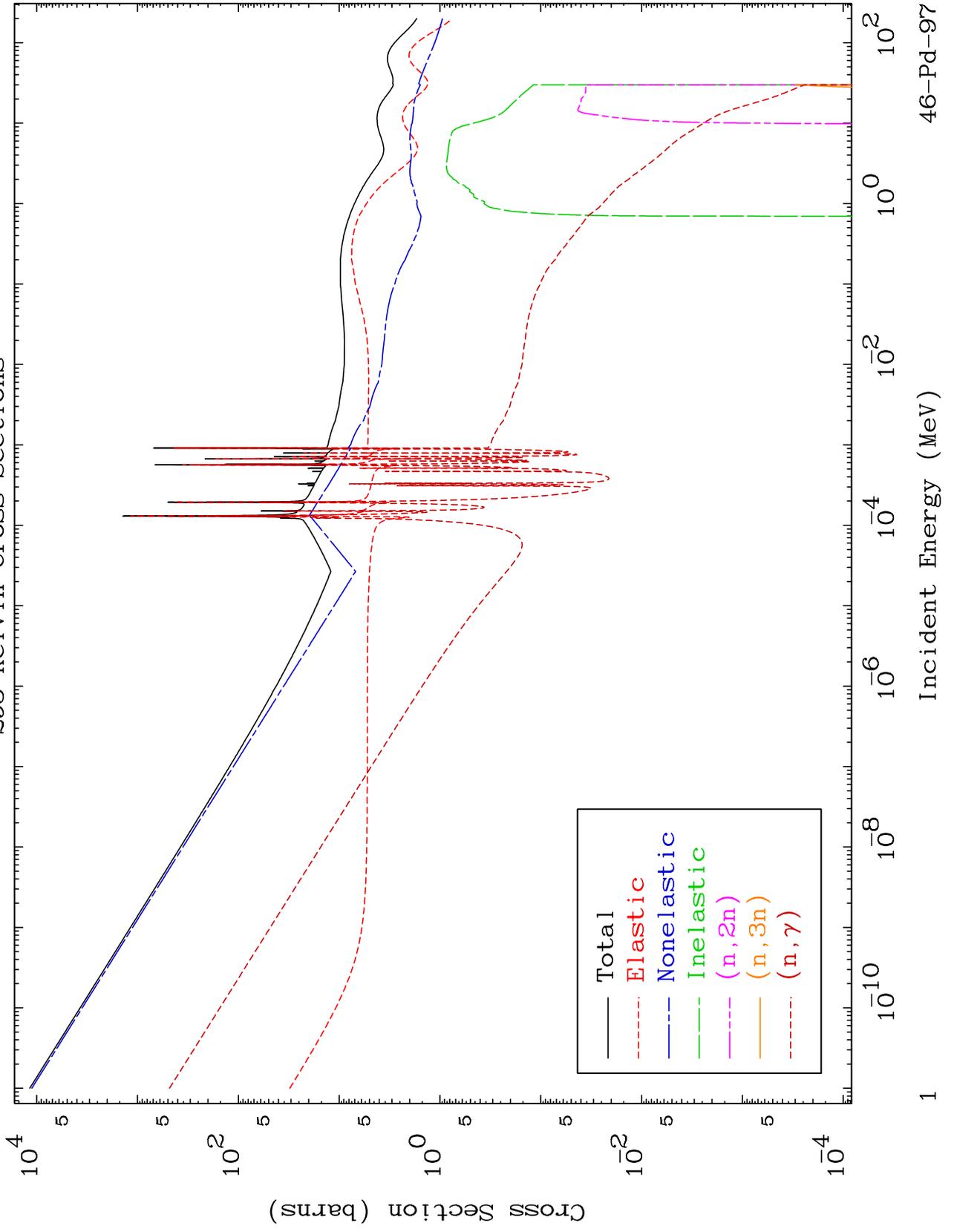
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Press Mouse Button to Start

MAT 4610

Neutron Major
293 Kelvin Cross Sections

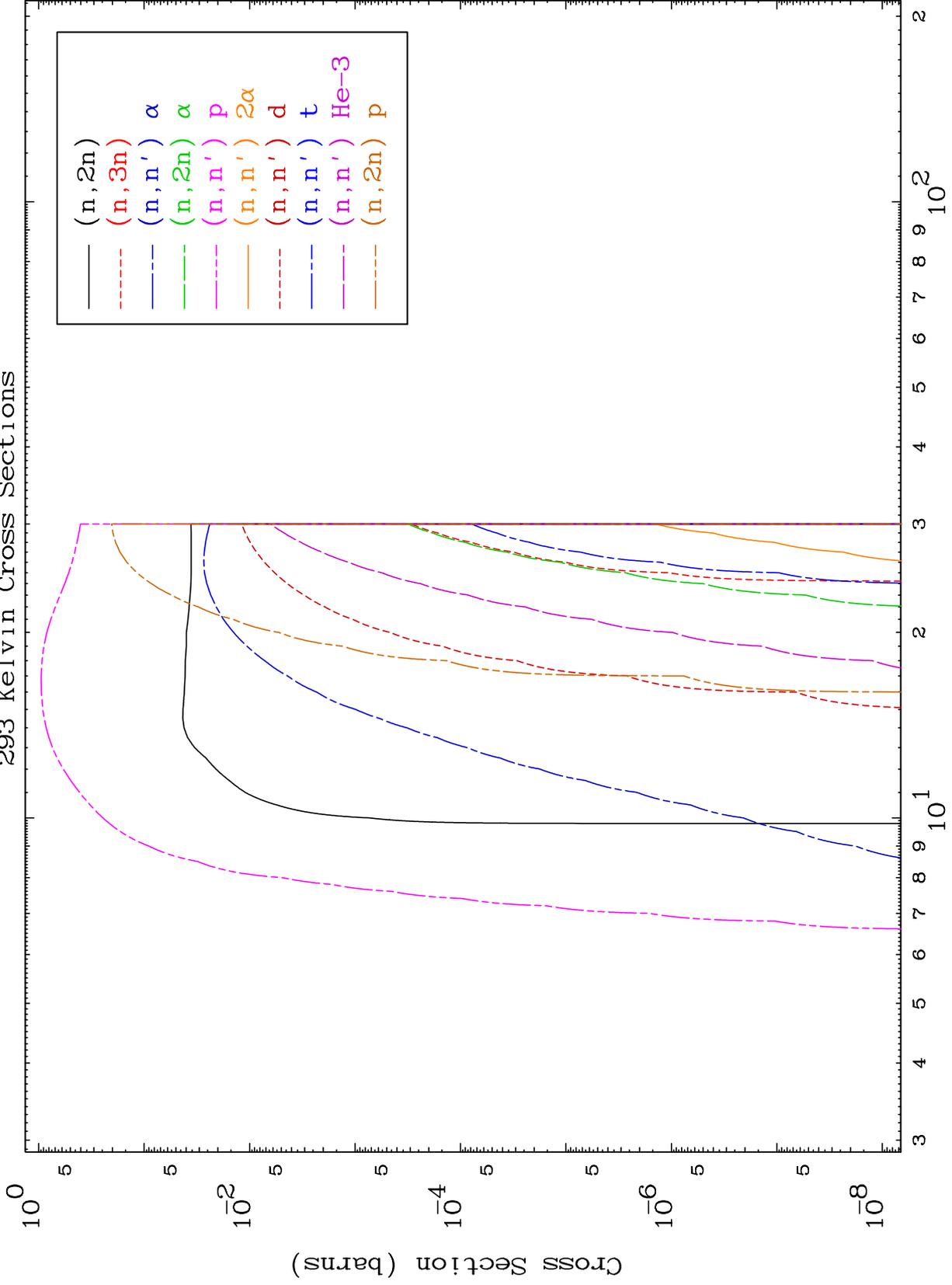
46-Pd-97



MAT 4610

Neutron Absorption
293 Kelvin Cross Sections

46-Pd-97



2

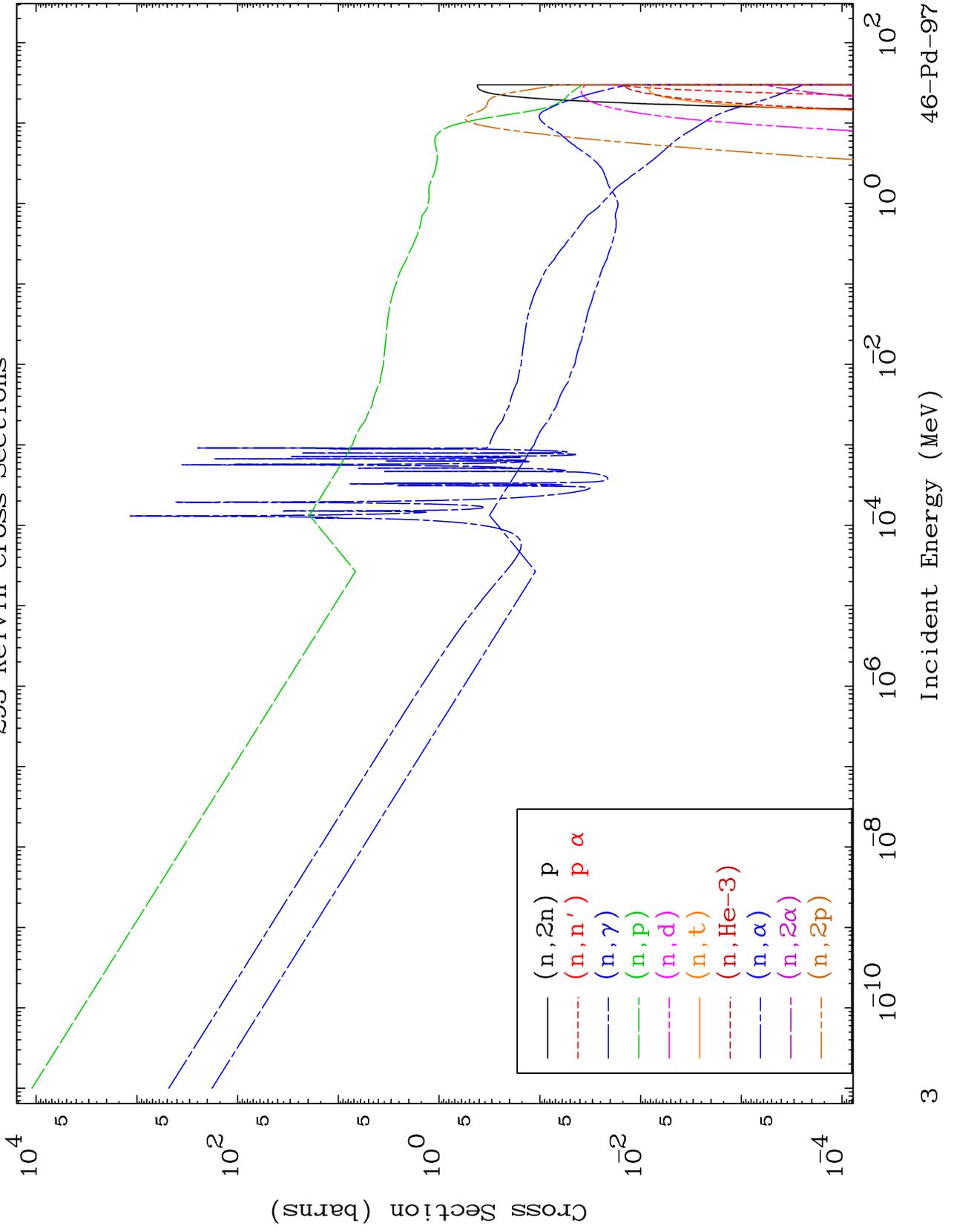
Incident Energy (MeV)

46-Pd-97

MAT 4610

Neutron Absorption
293 Kelvin Cross Sections

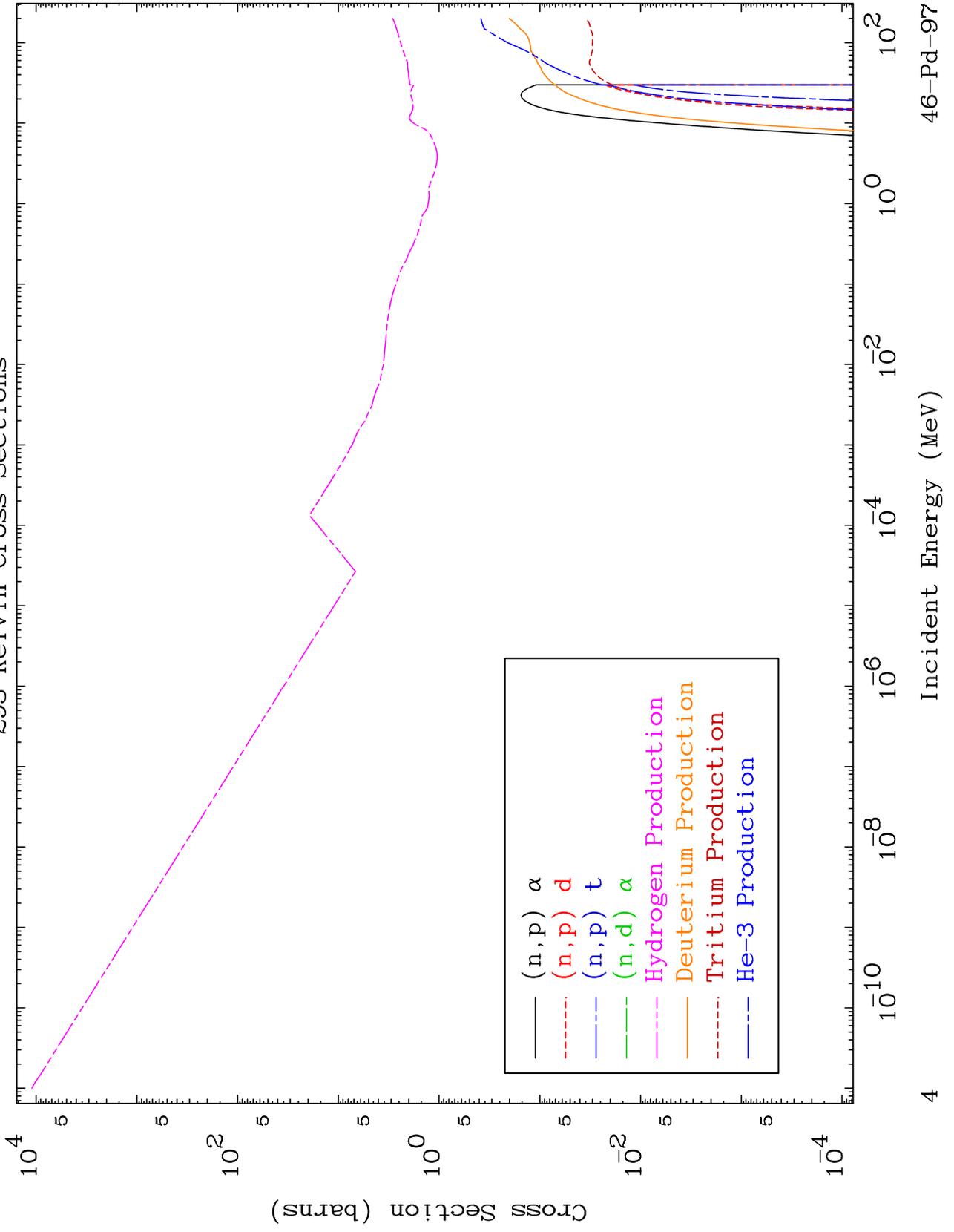
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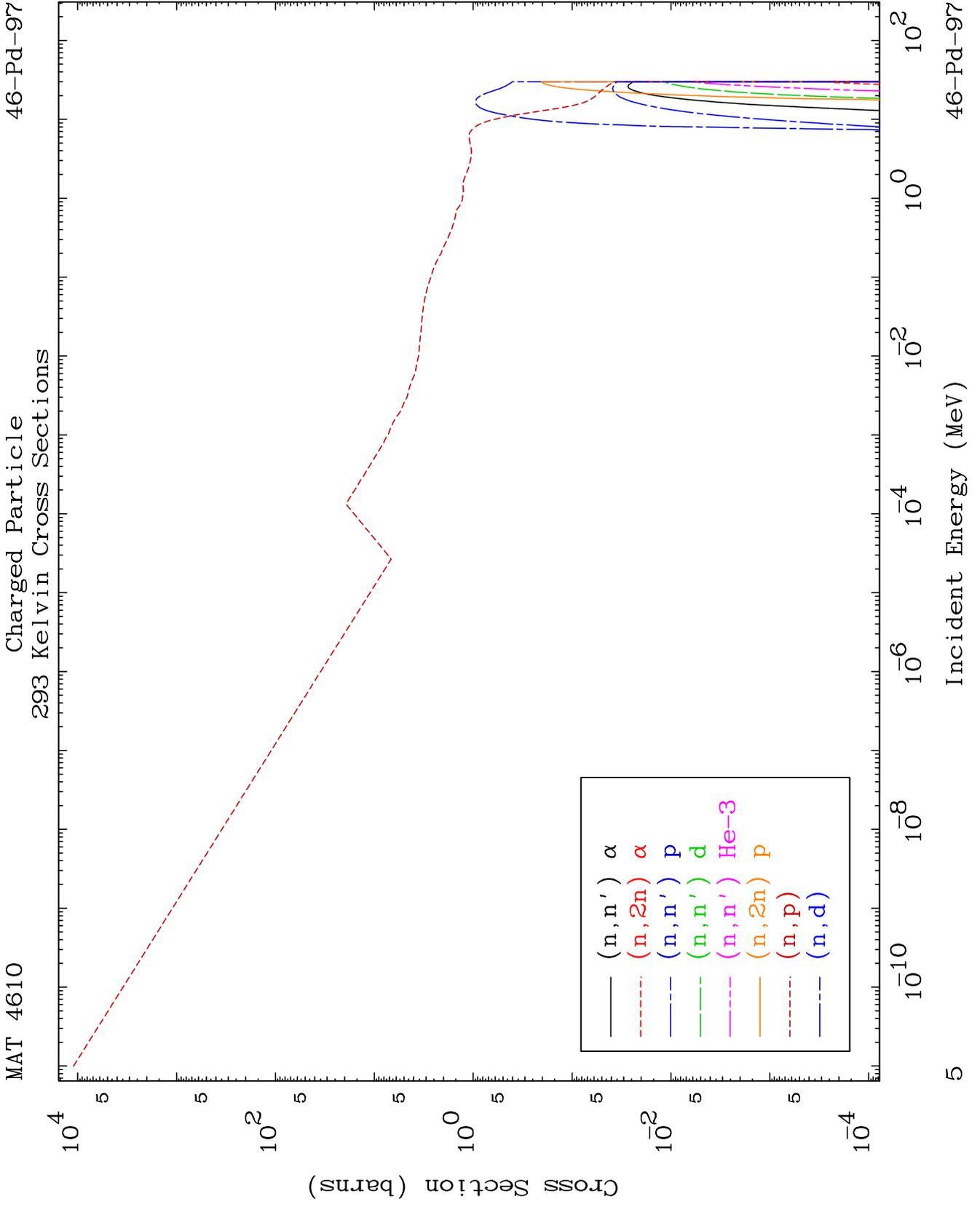


MAT 4610

Neutron Absorption
293 Kelvin Cross Sections

46-Pd-97

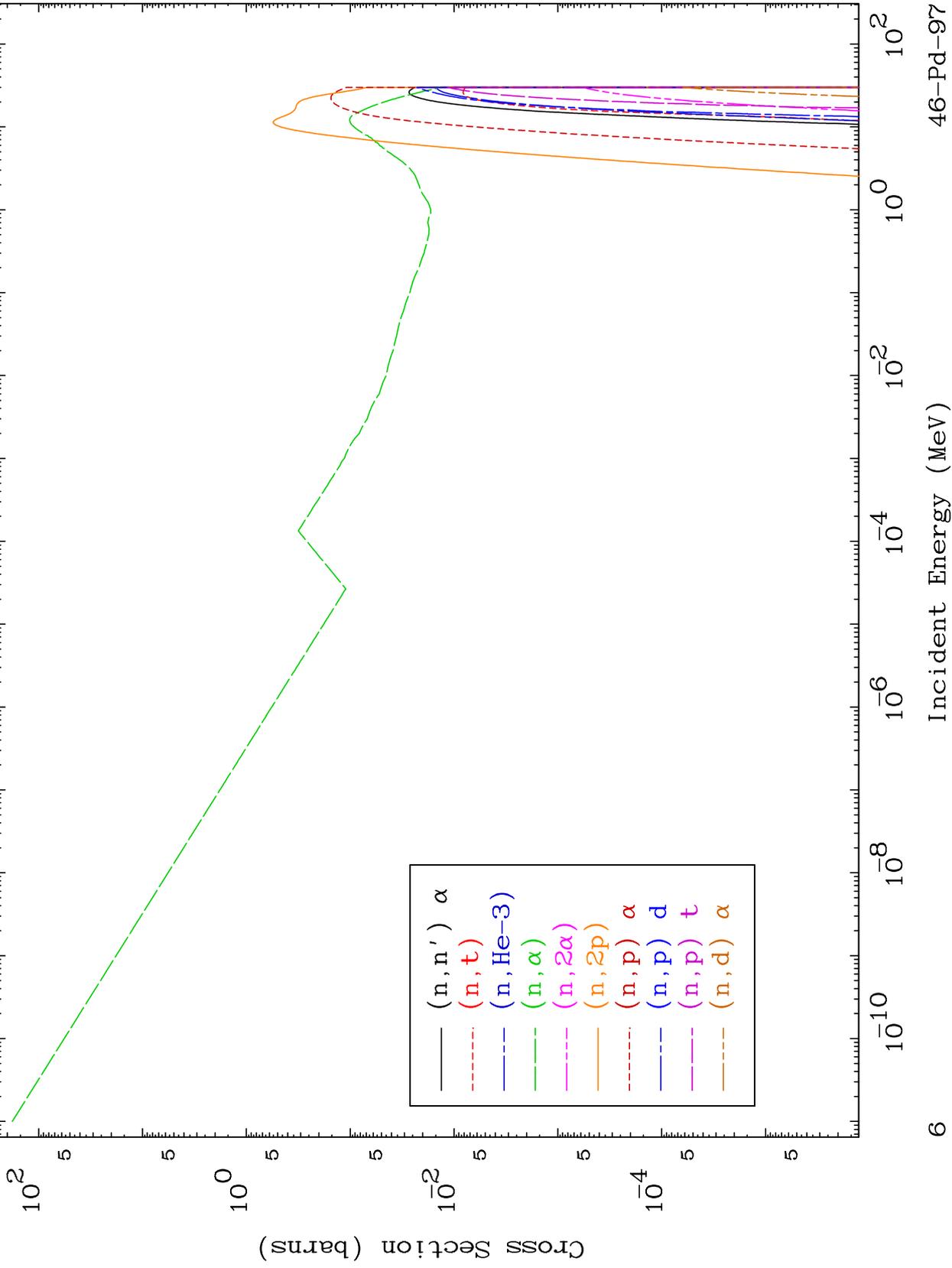




MAT 4610

Charged Particle
293 Kelvin Cross Sections

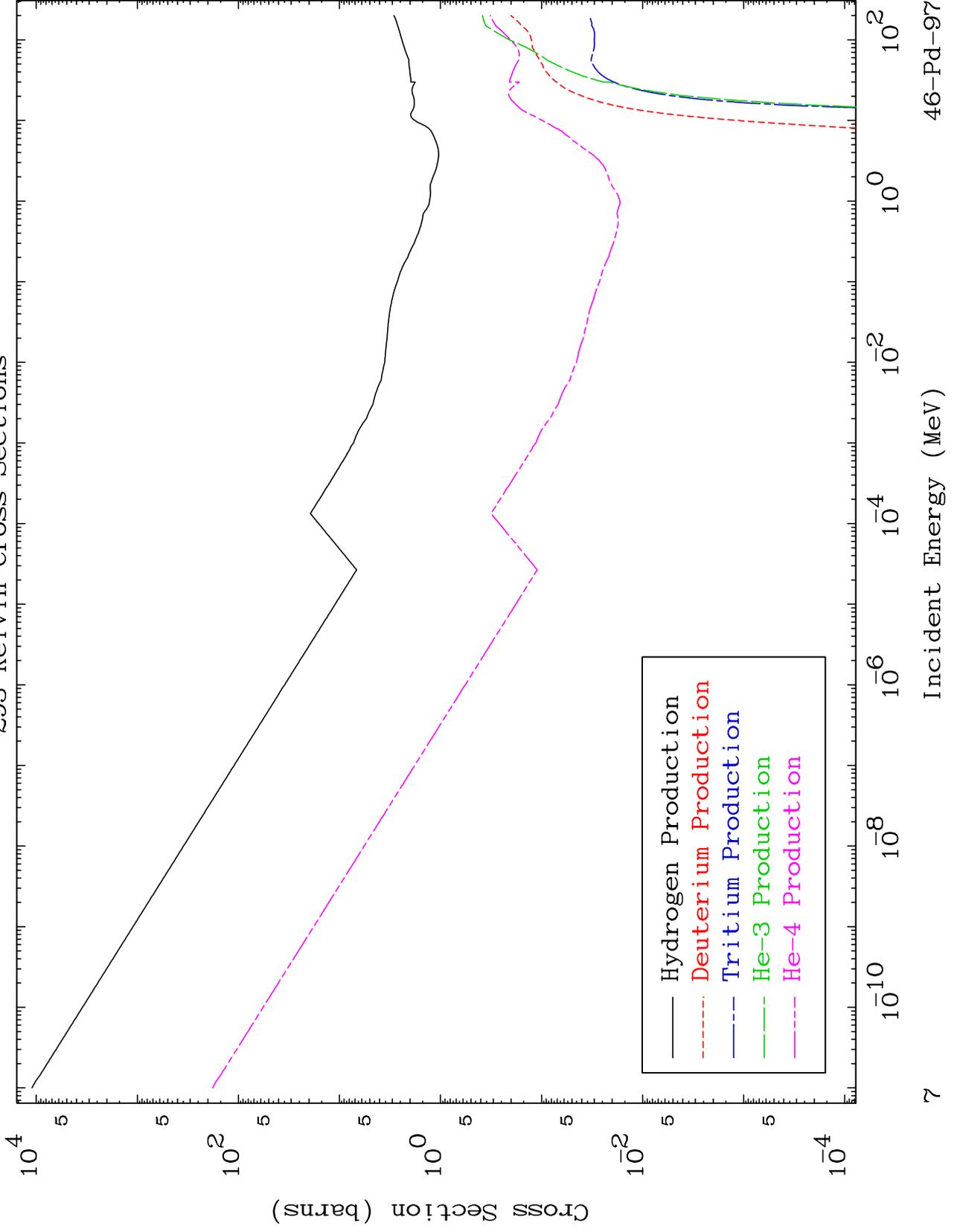
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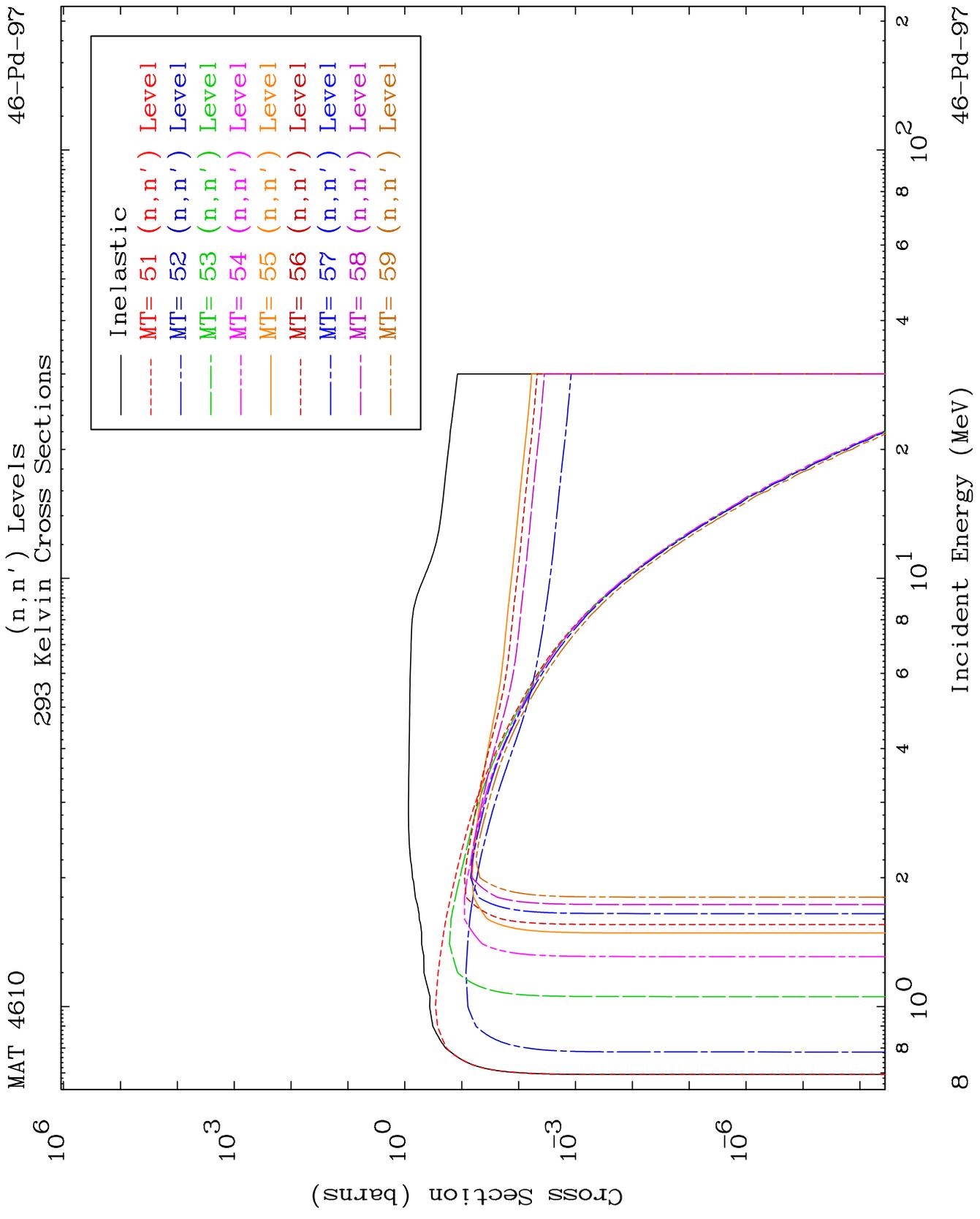


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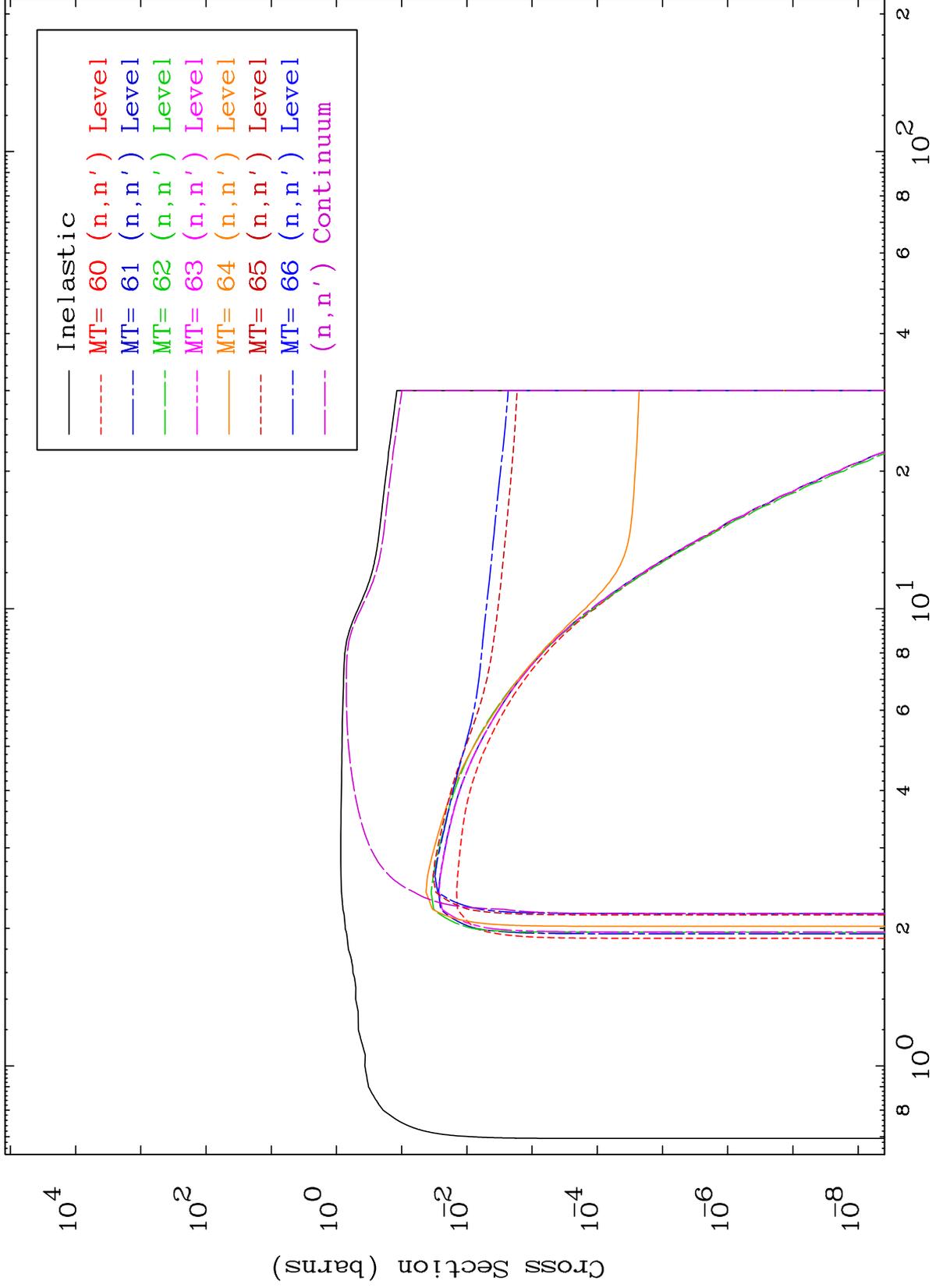
Particle Production
293 Kelvin Cross Sections

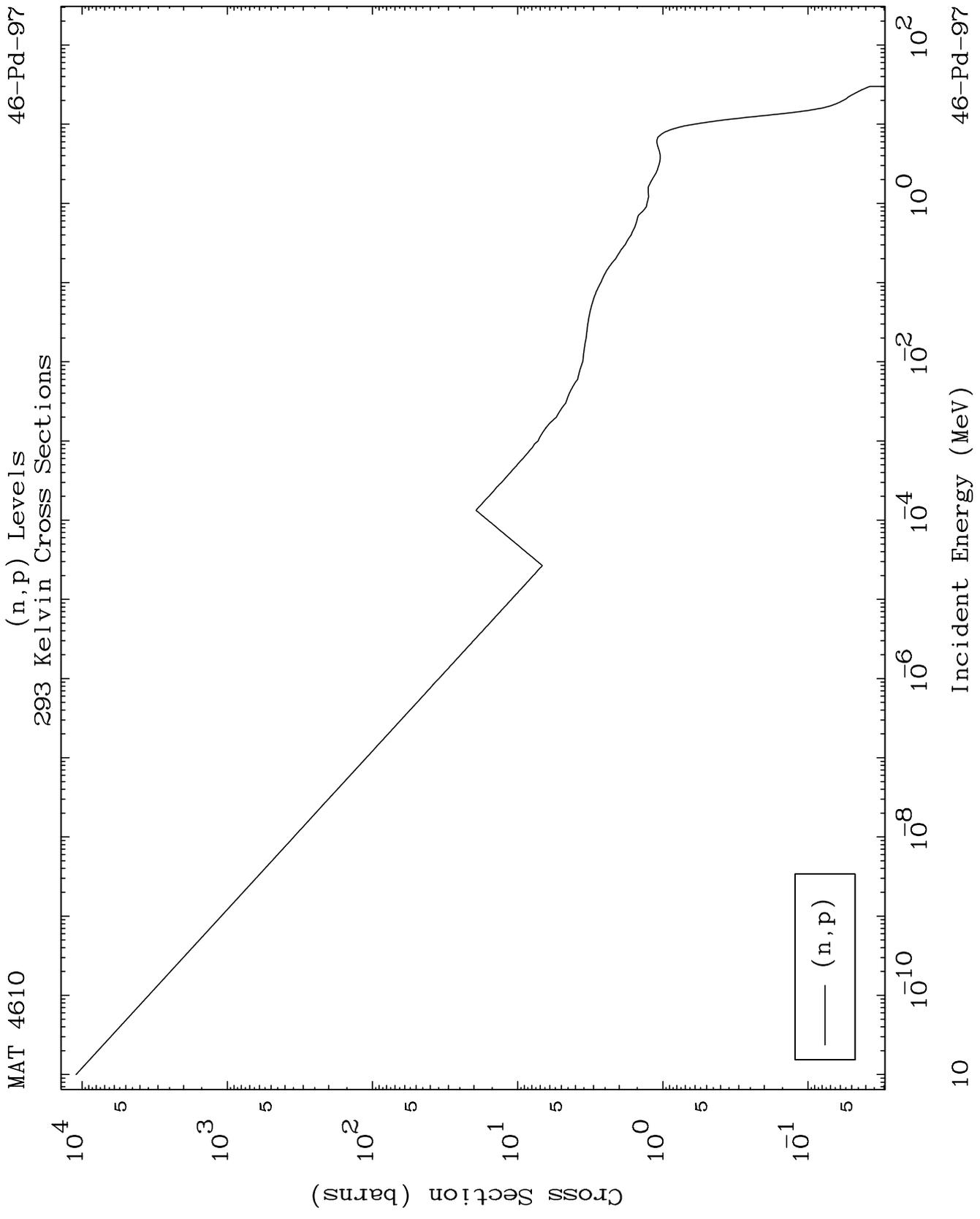
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293 Kelvin Cross Sections

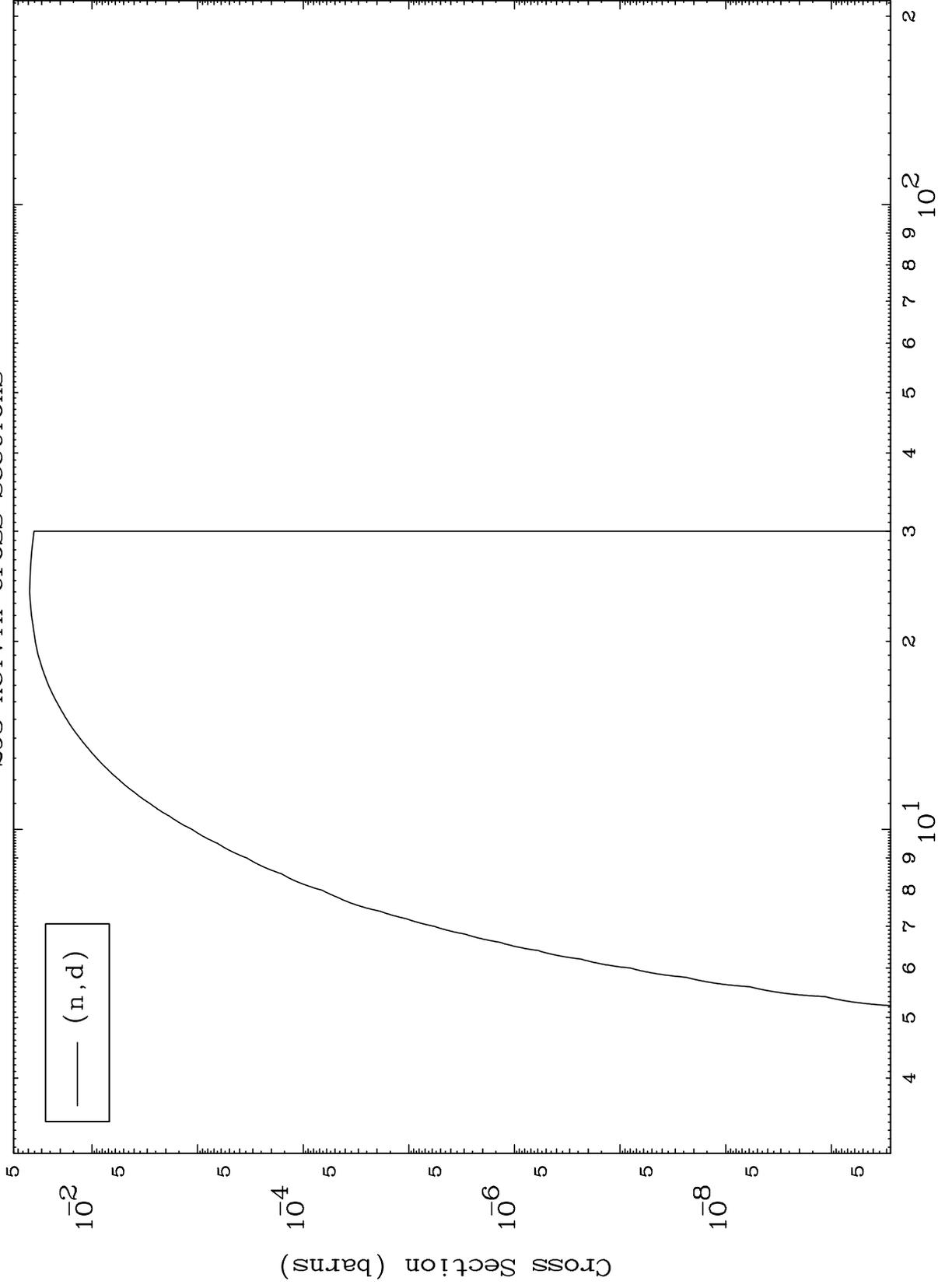




MAT 4610

(n,d) Levels
293 Kelvin Cross Sections

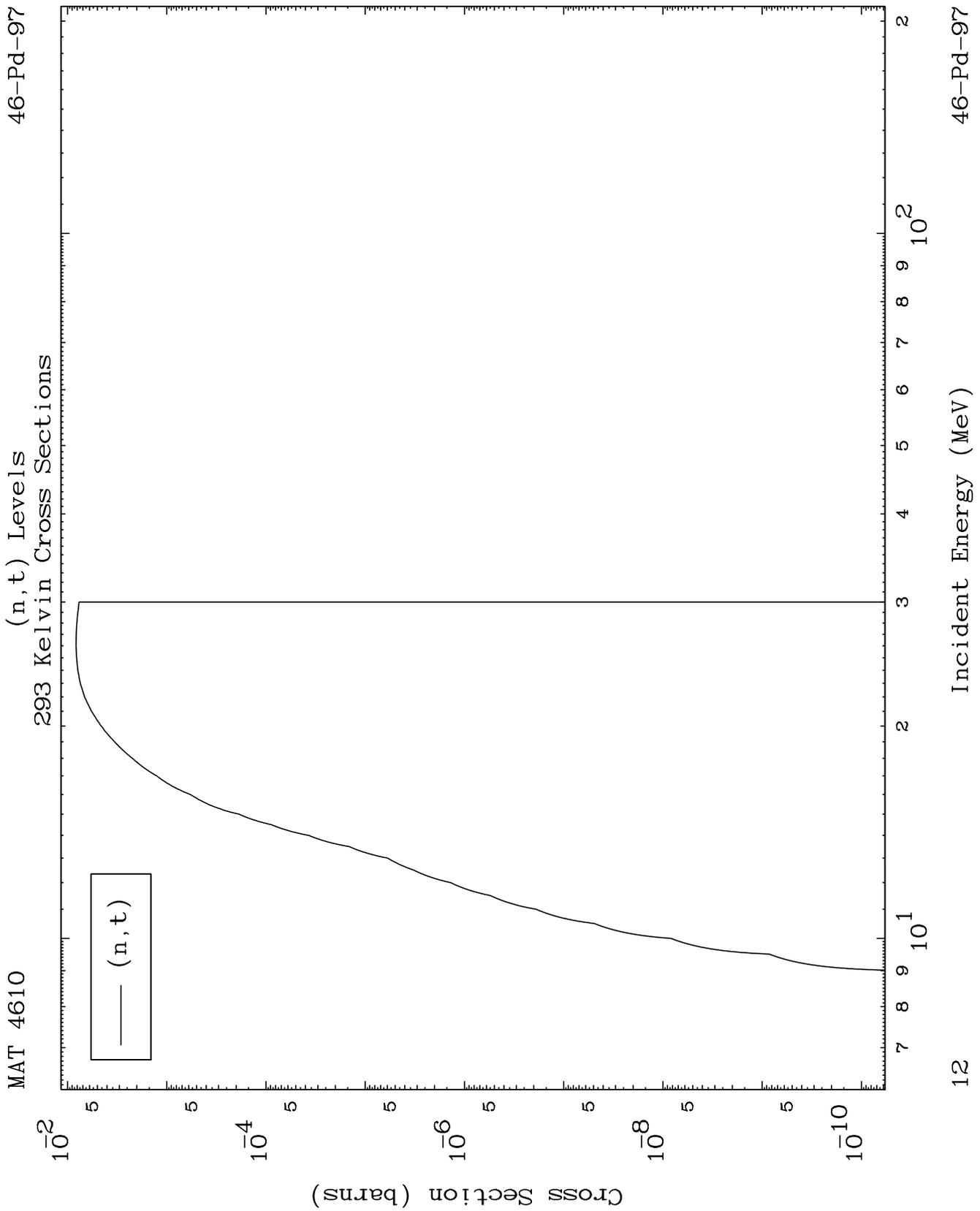
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11

Incident Energy (MeV)

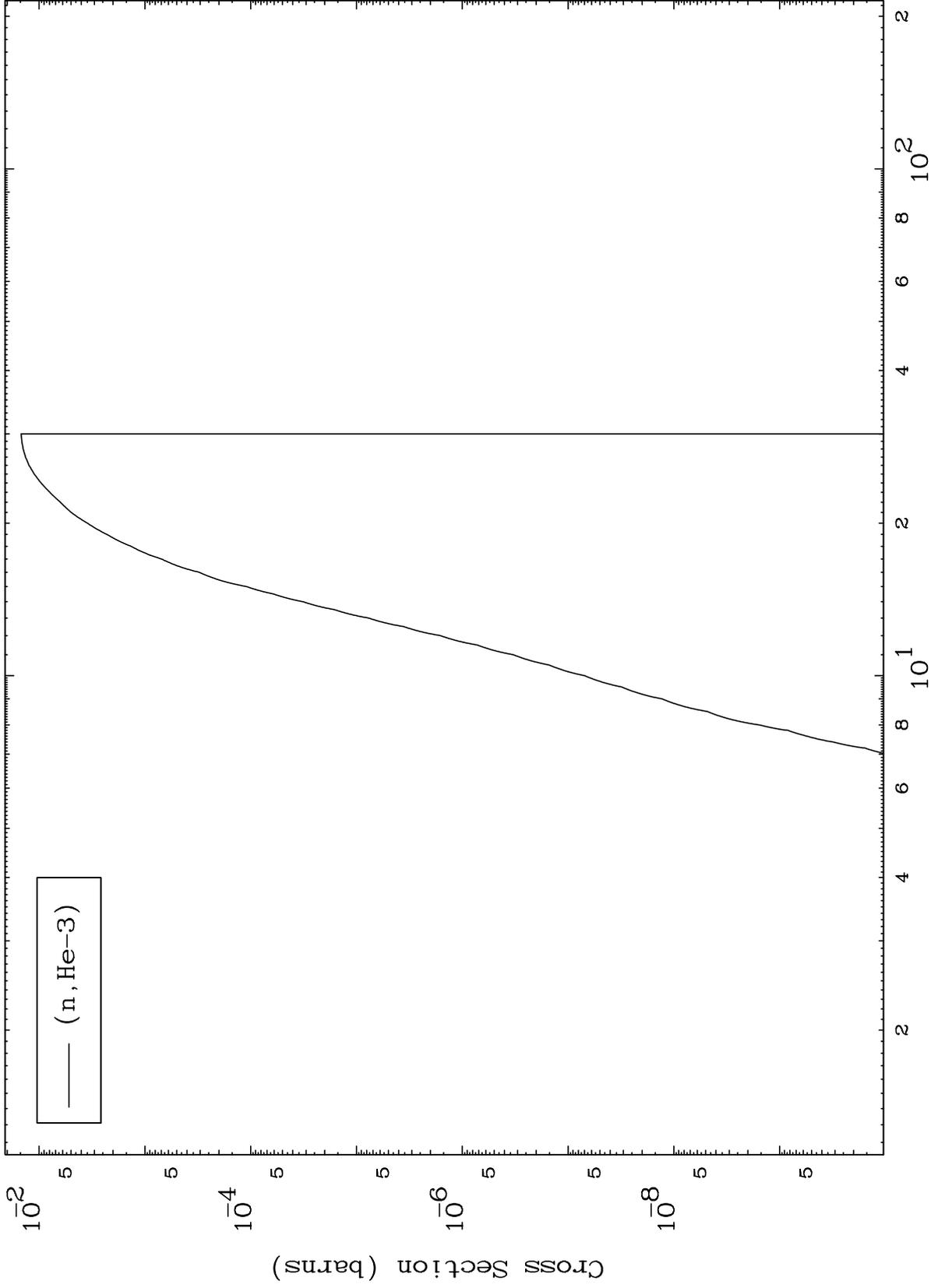
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MAT 4610

(n,He3) Levels
293 Kelvin Cross Sections

46-Pd-97



Incident Energy (MeV)

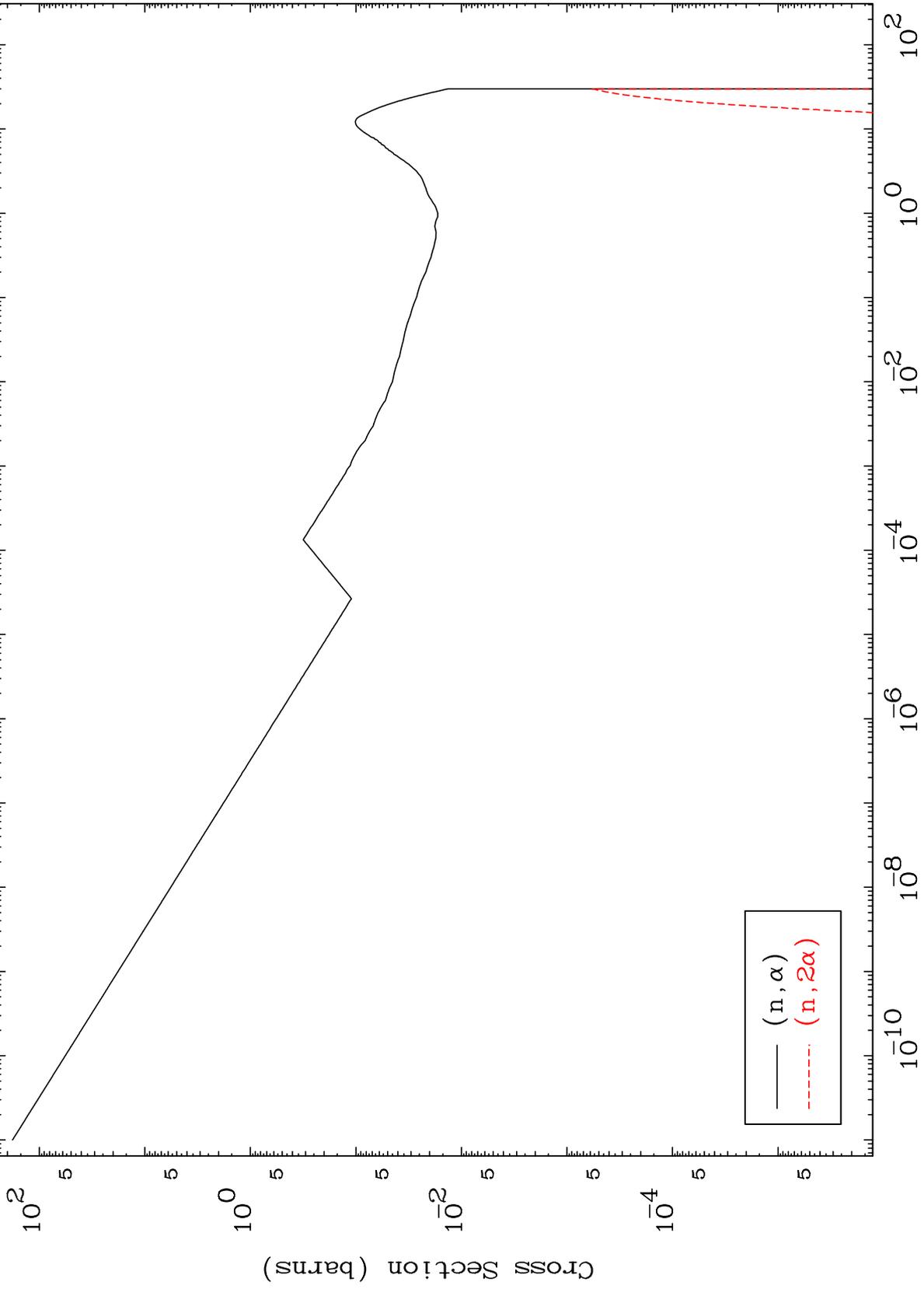
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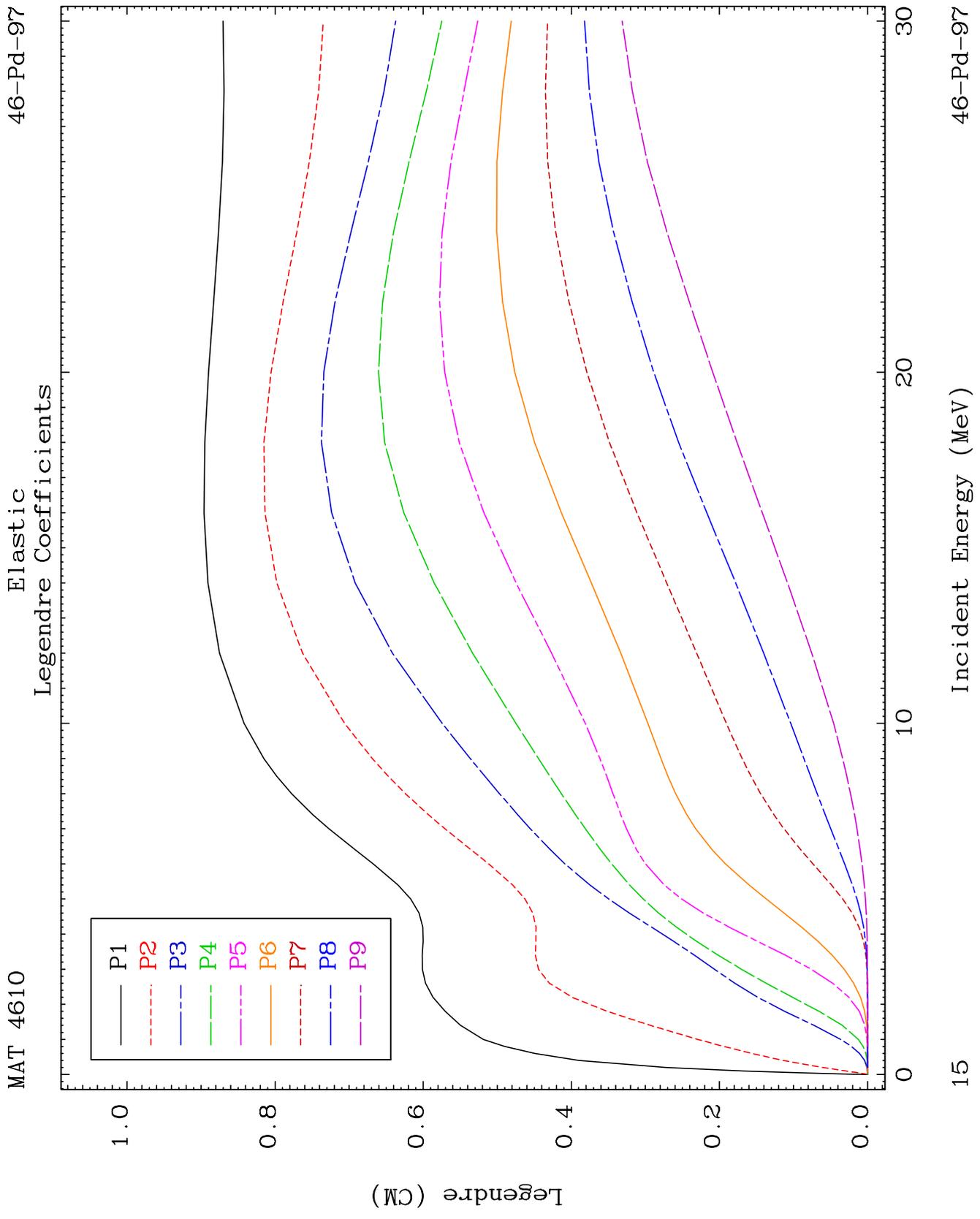
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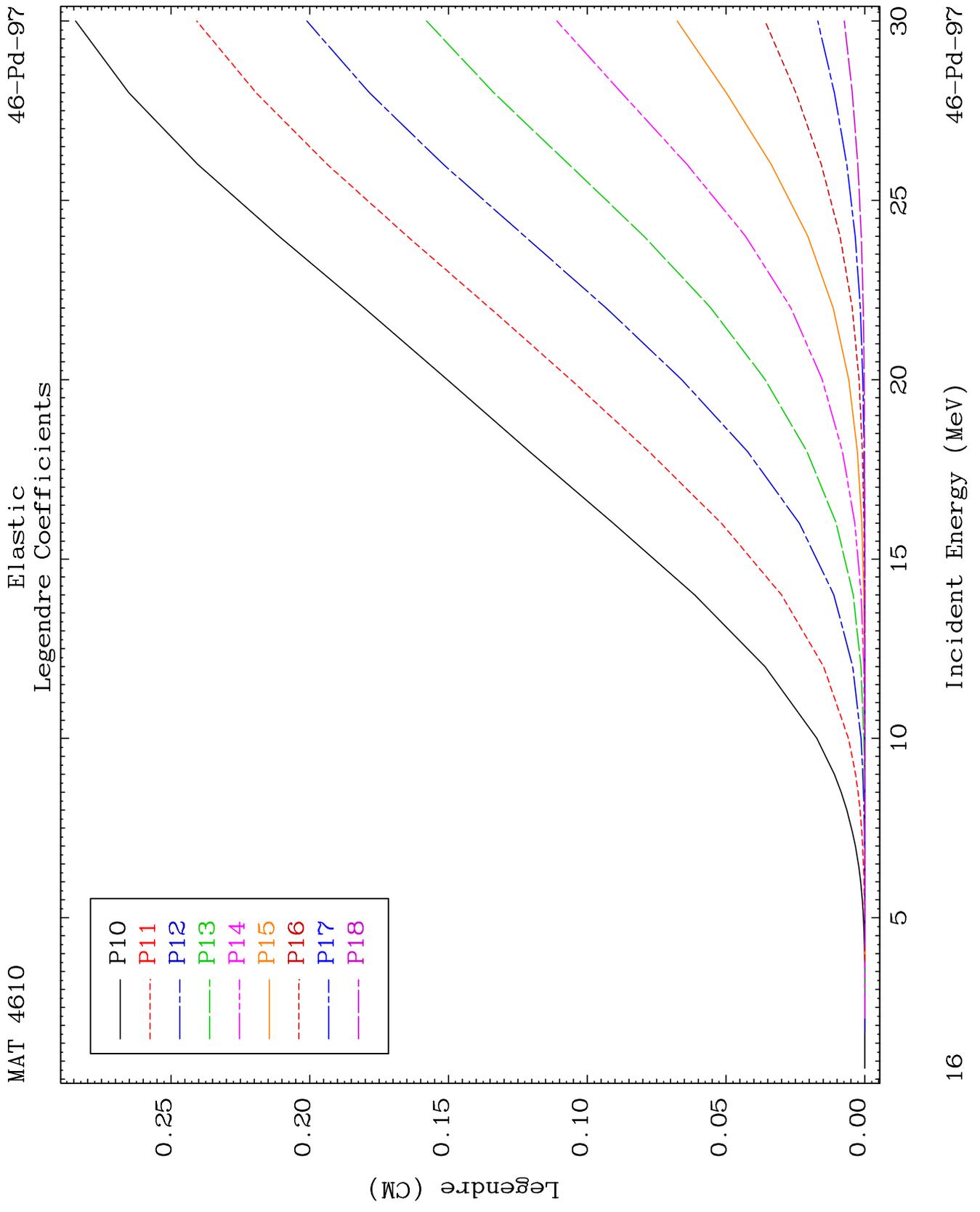
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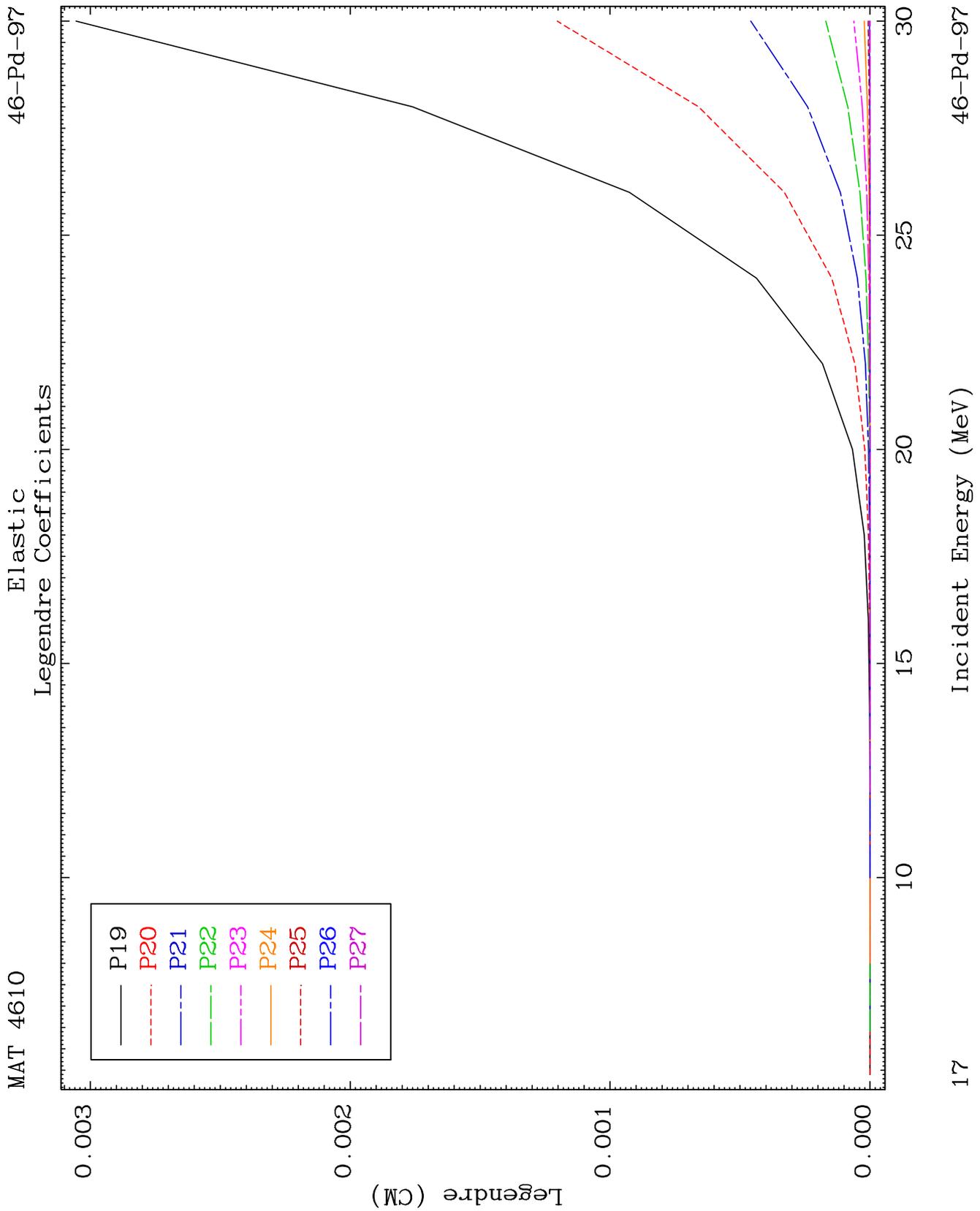
(n,α) Levels
293 Kelvin Cross Sections

46-Pd-97







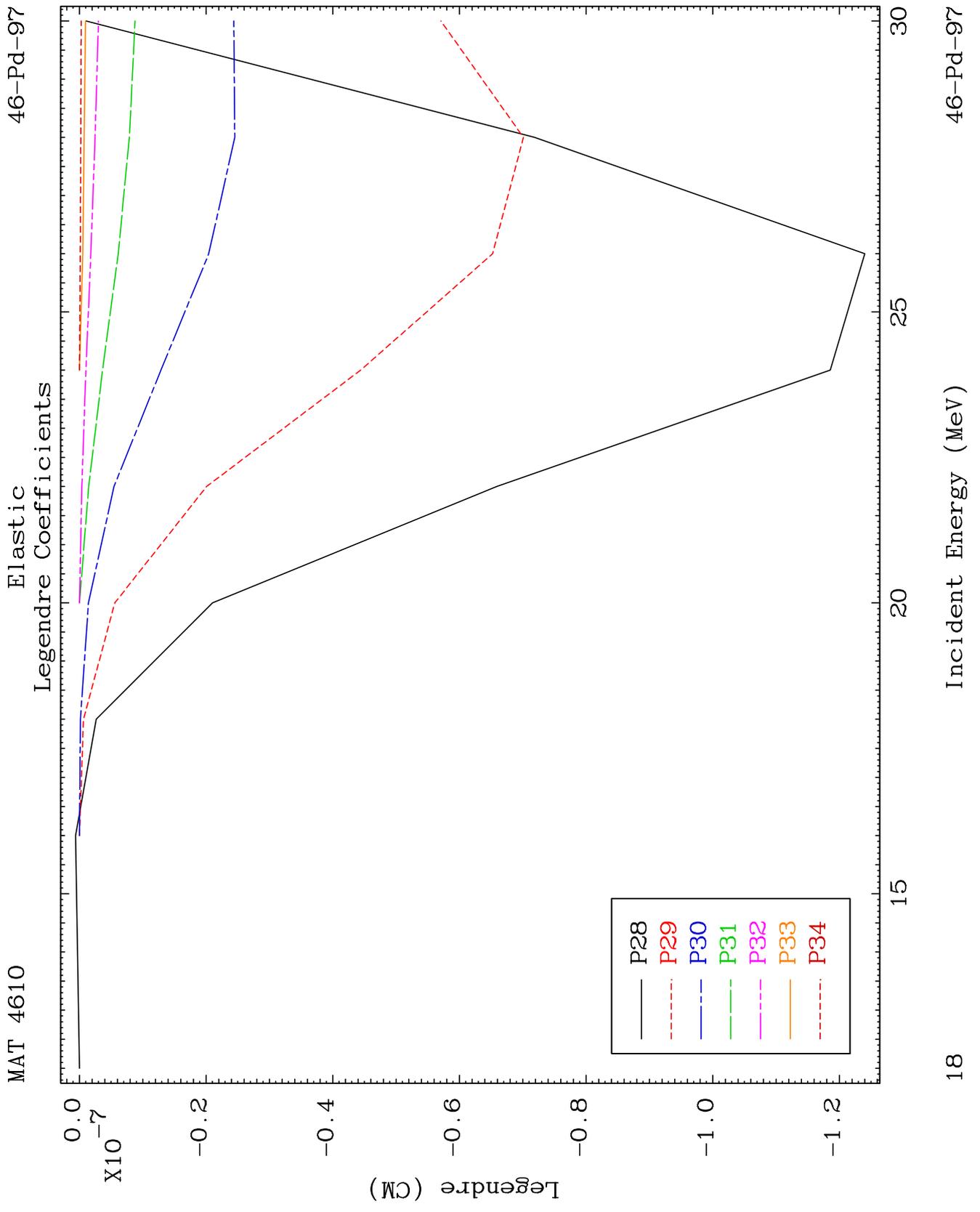


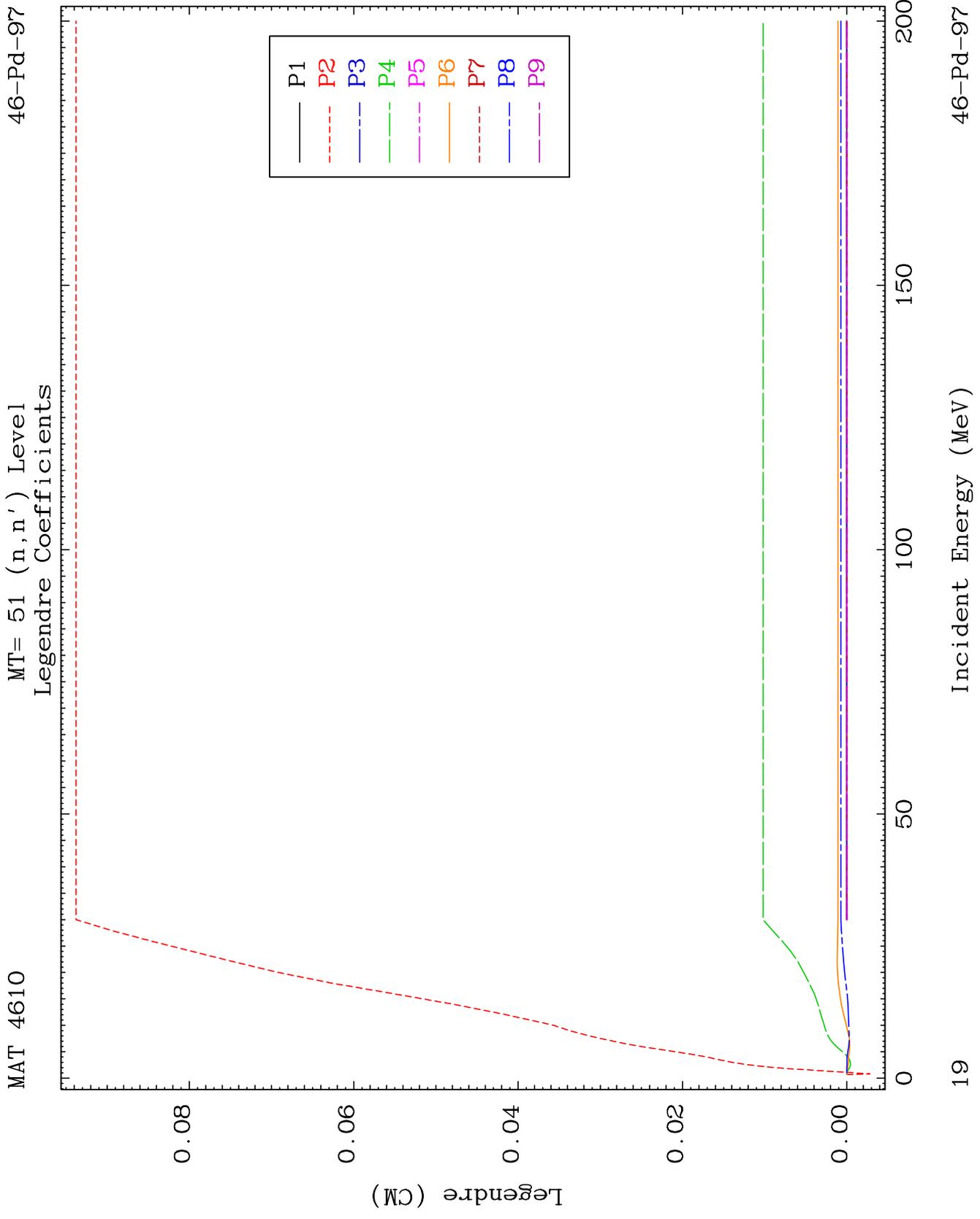
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17

46-Pd-97

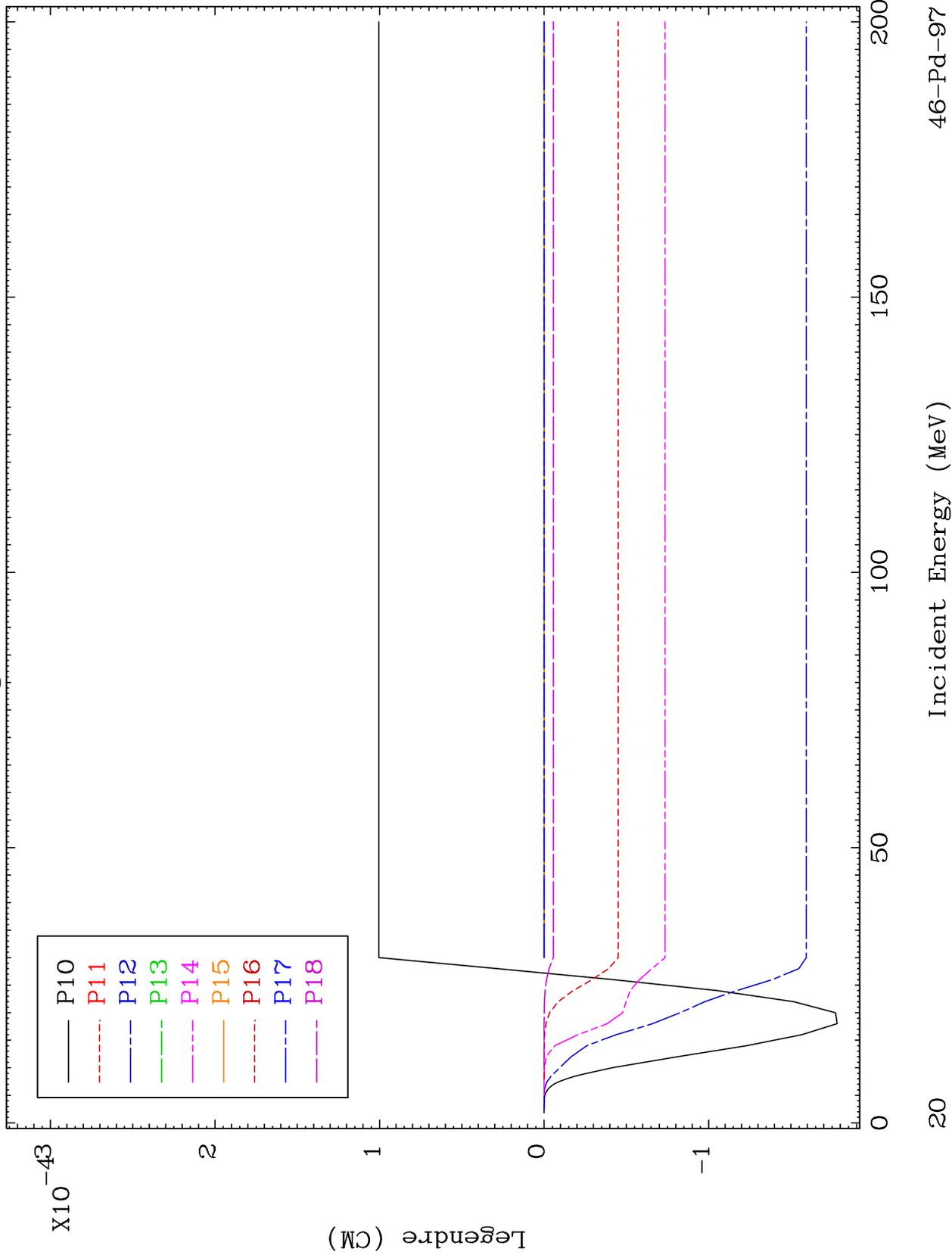




MAT 4610

MT= 51 (n,n') Level
Legendre Coefficients

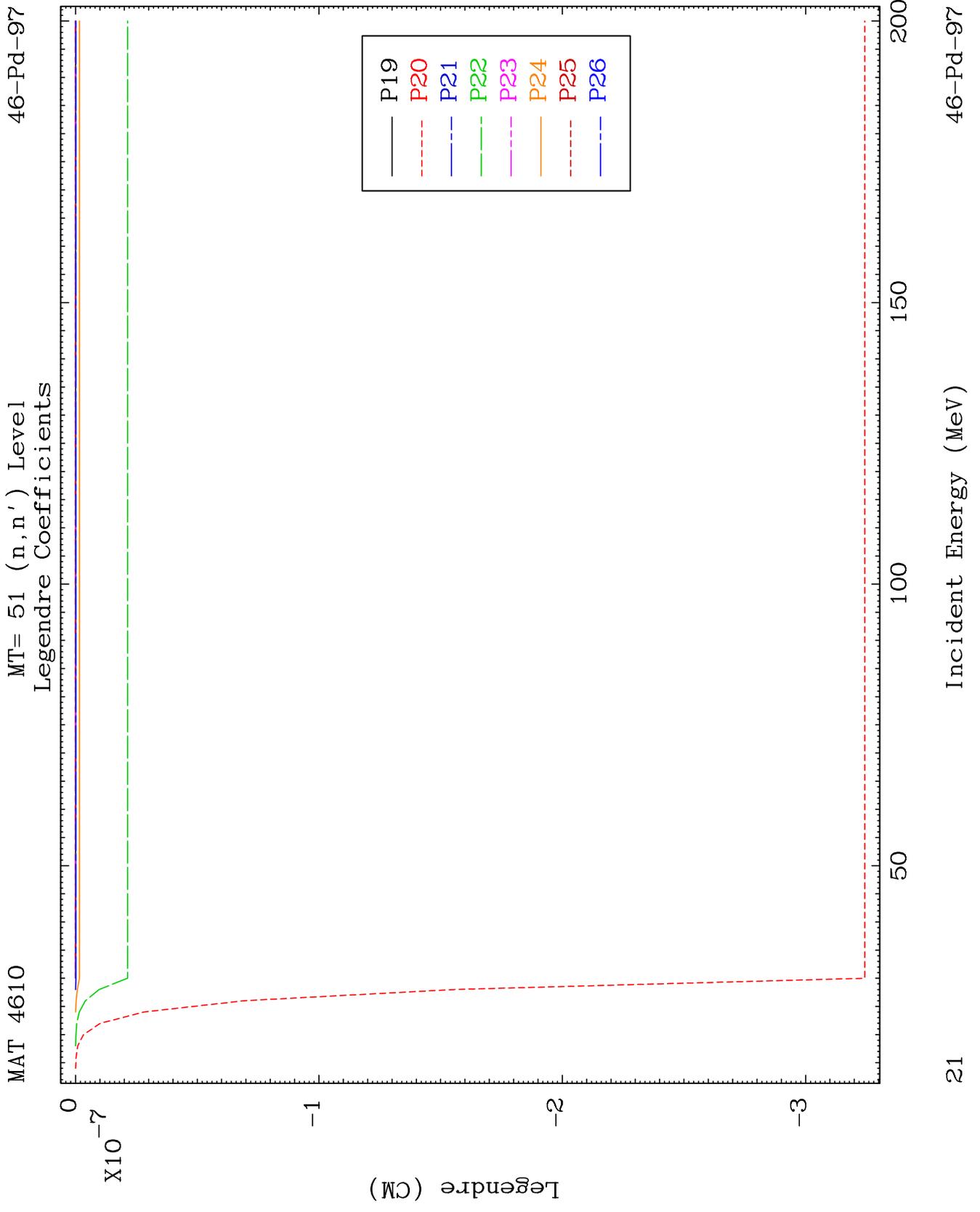
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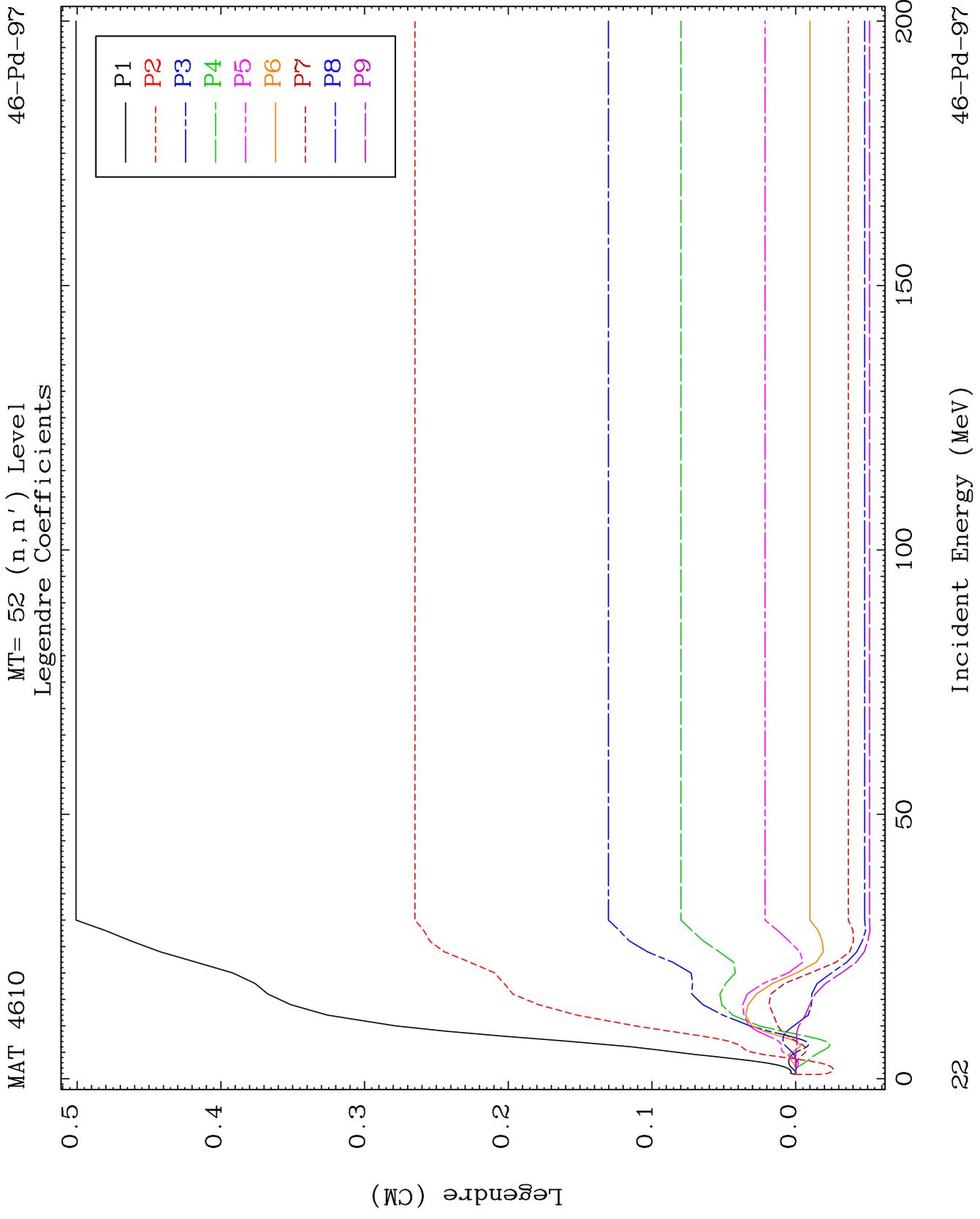


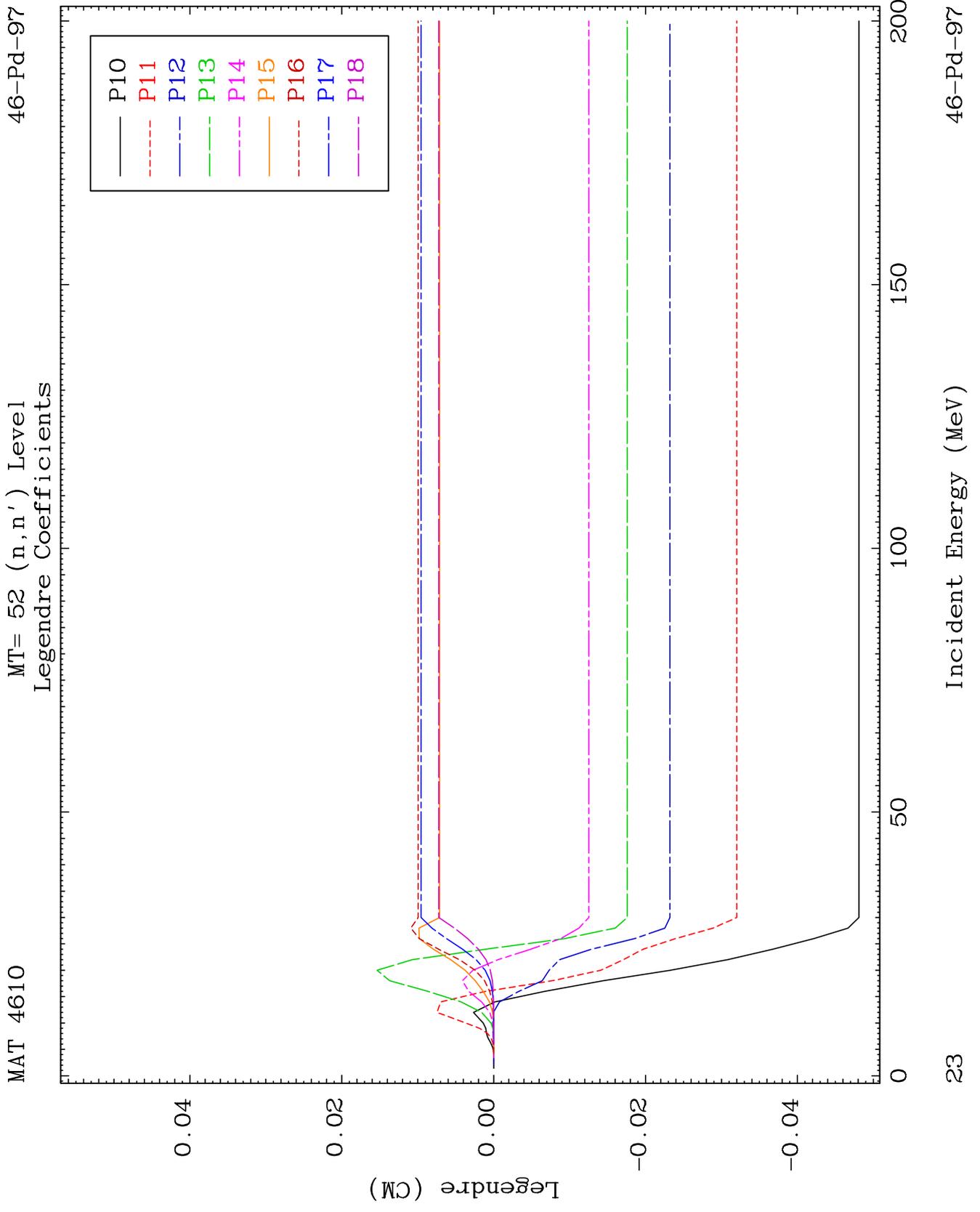
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Incident Energy (MeV)

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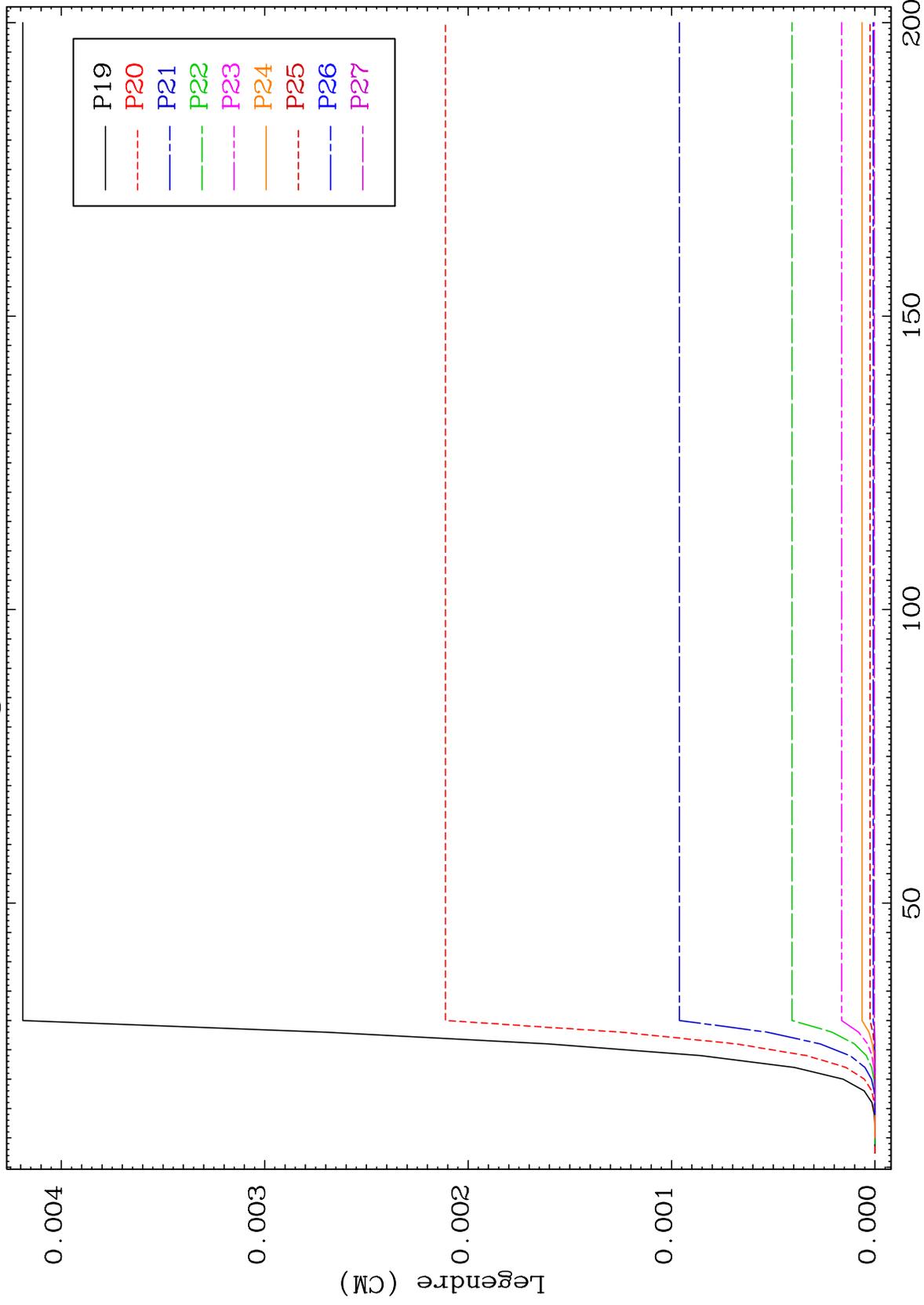




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Legendre Coefficients

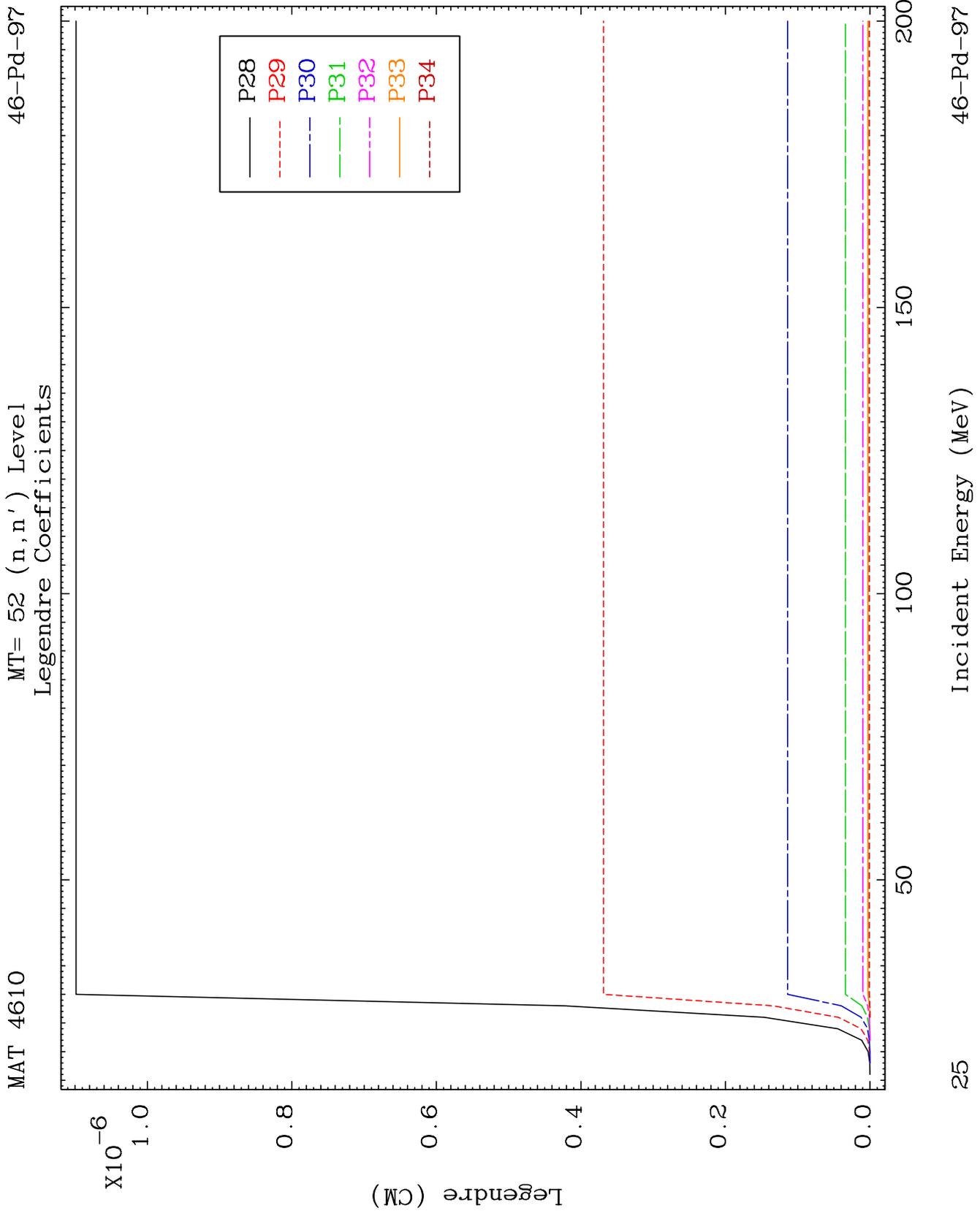
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24

Incident Energy (MeV)

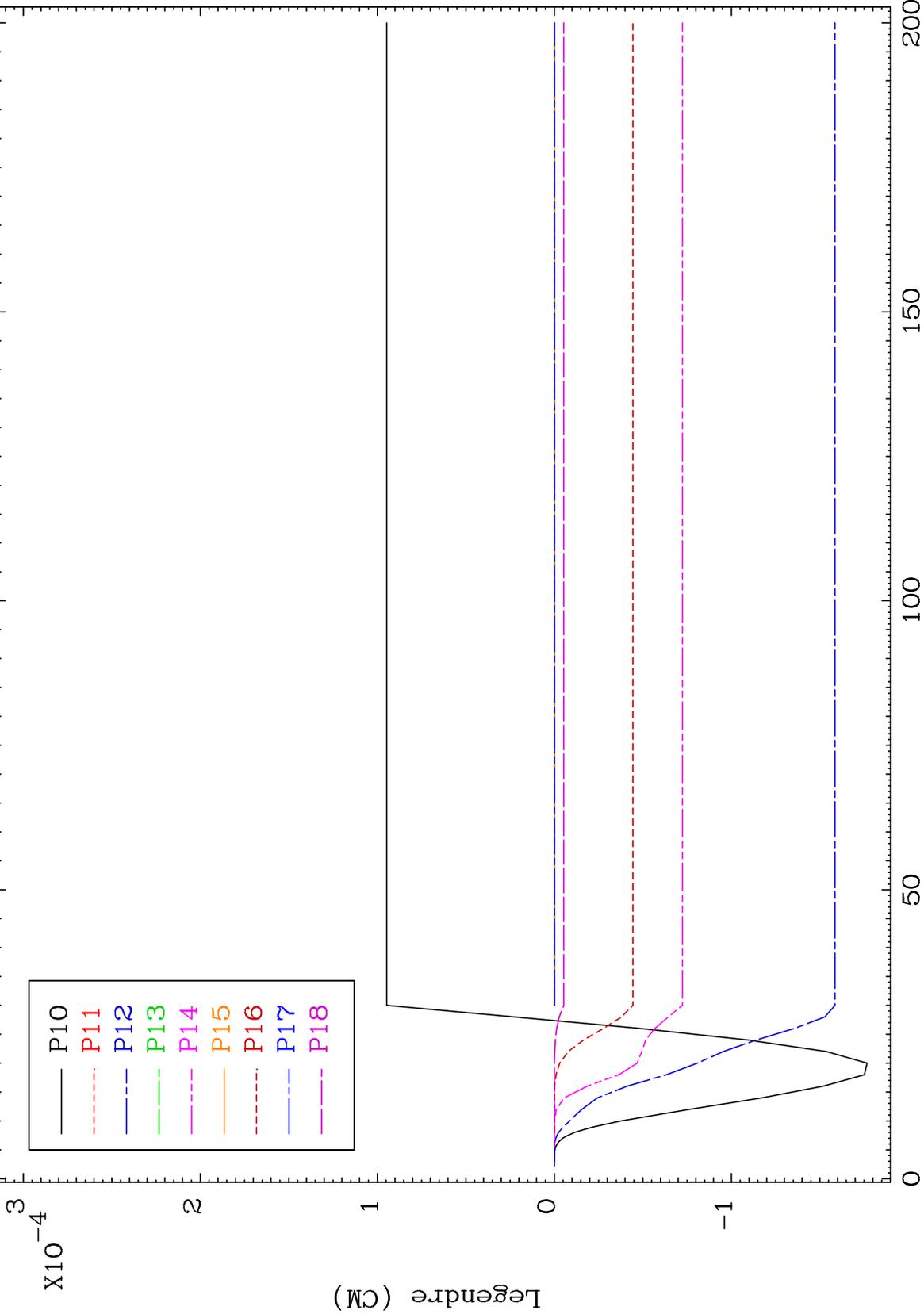
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MAT 4610

MT= 53 (n,n') Level
Legendre Coefficients

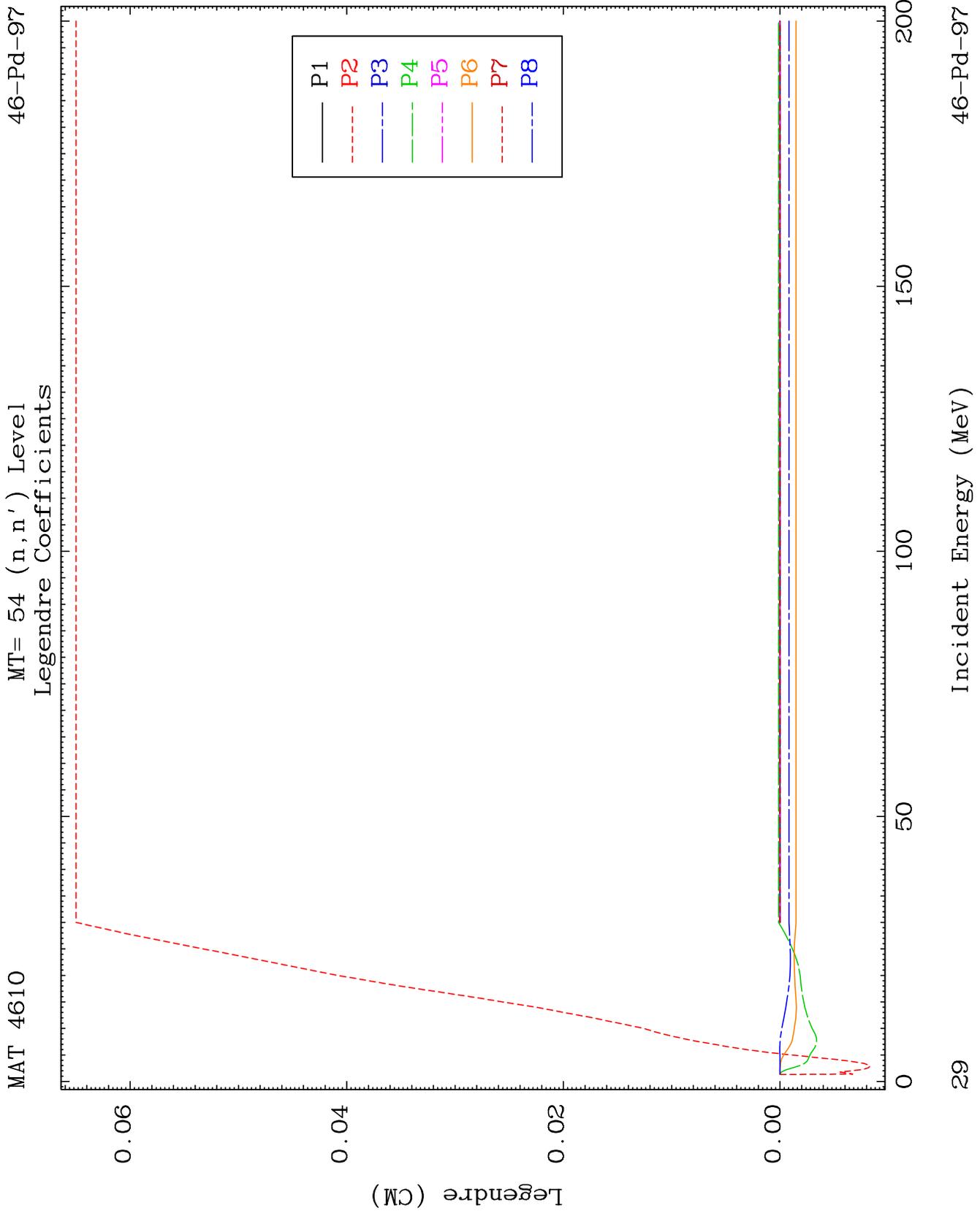
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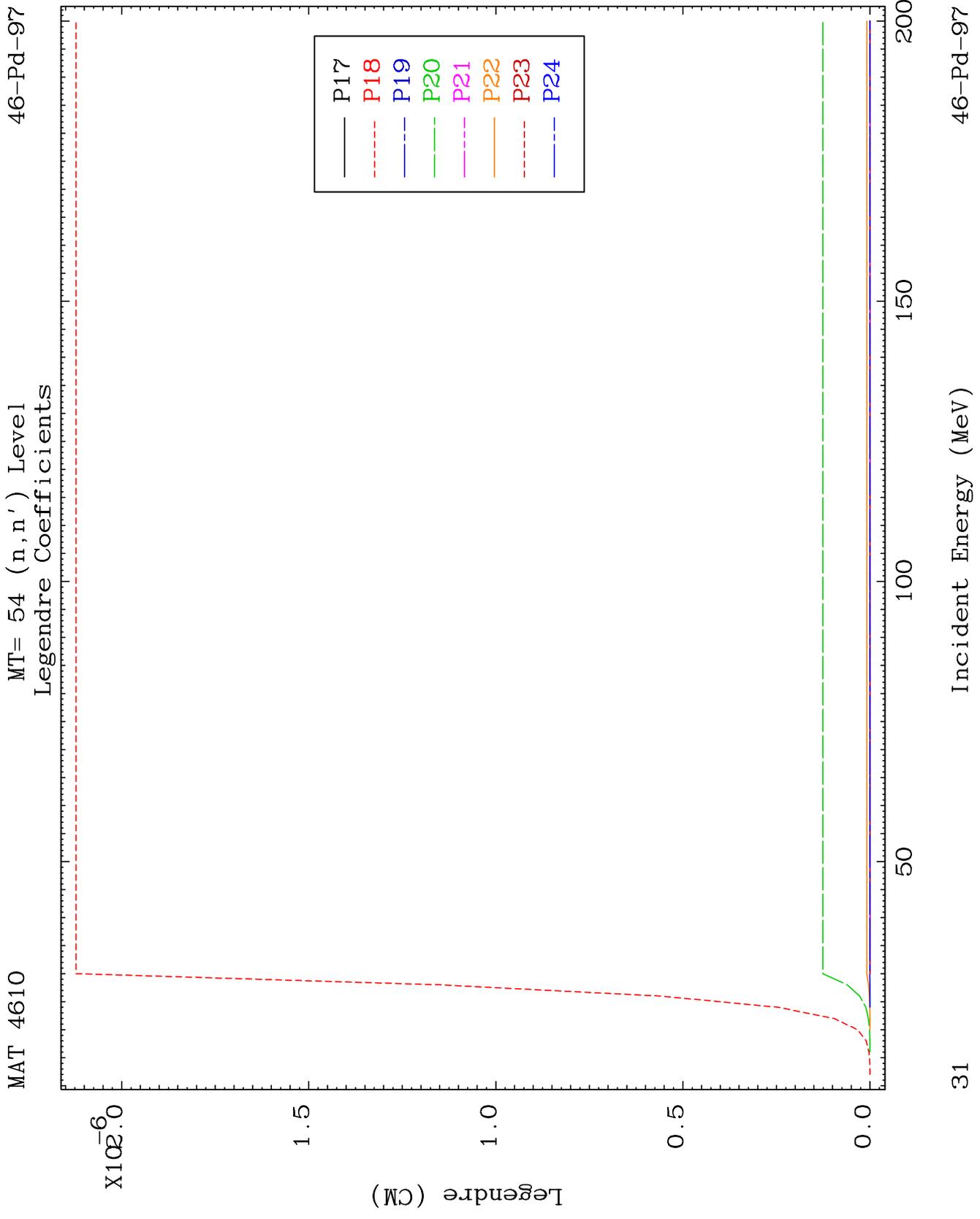


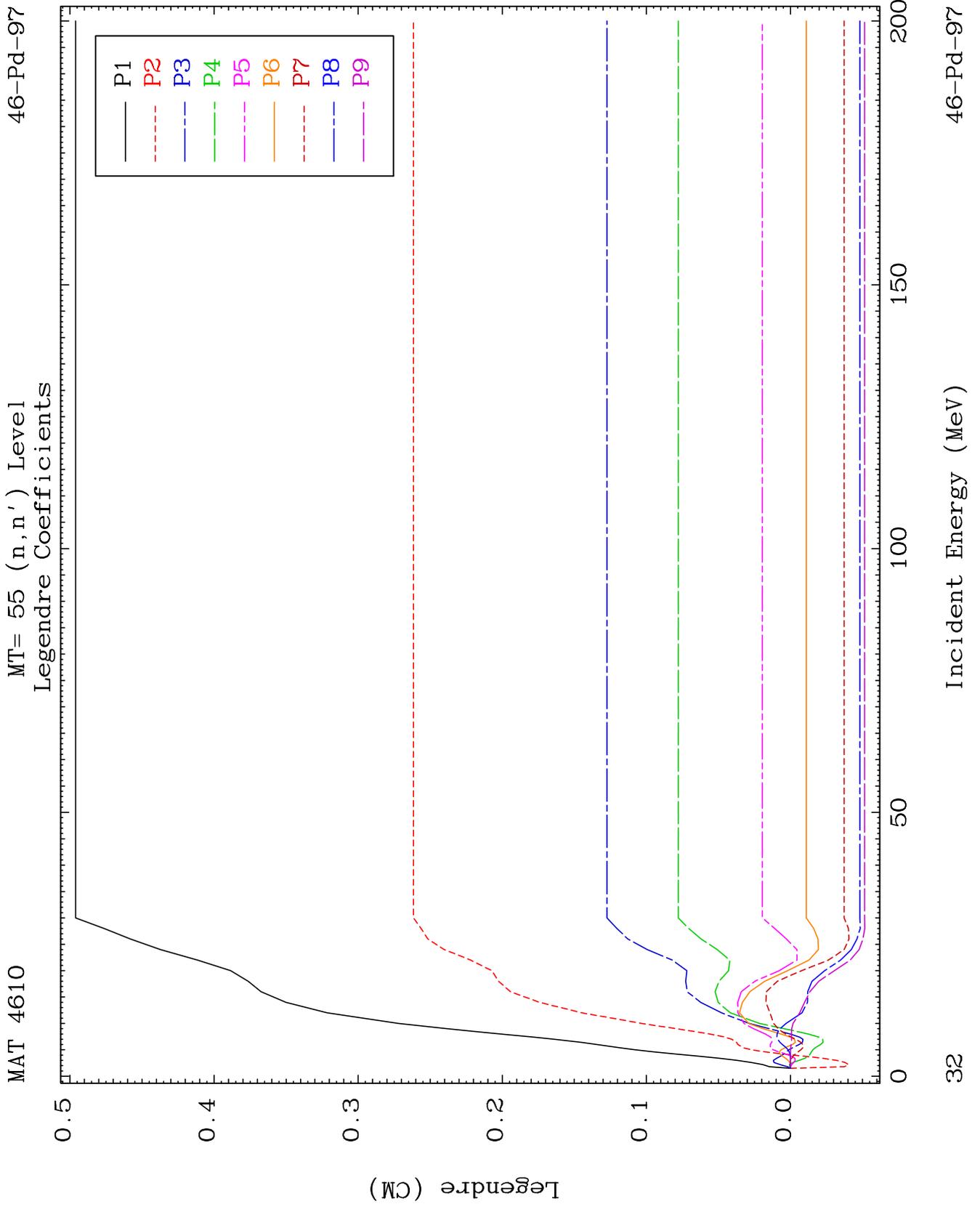
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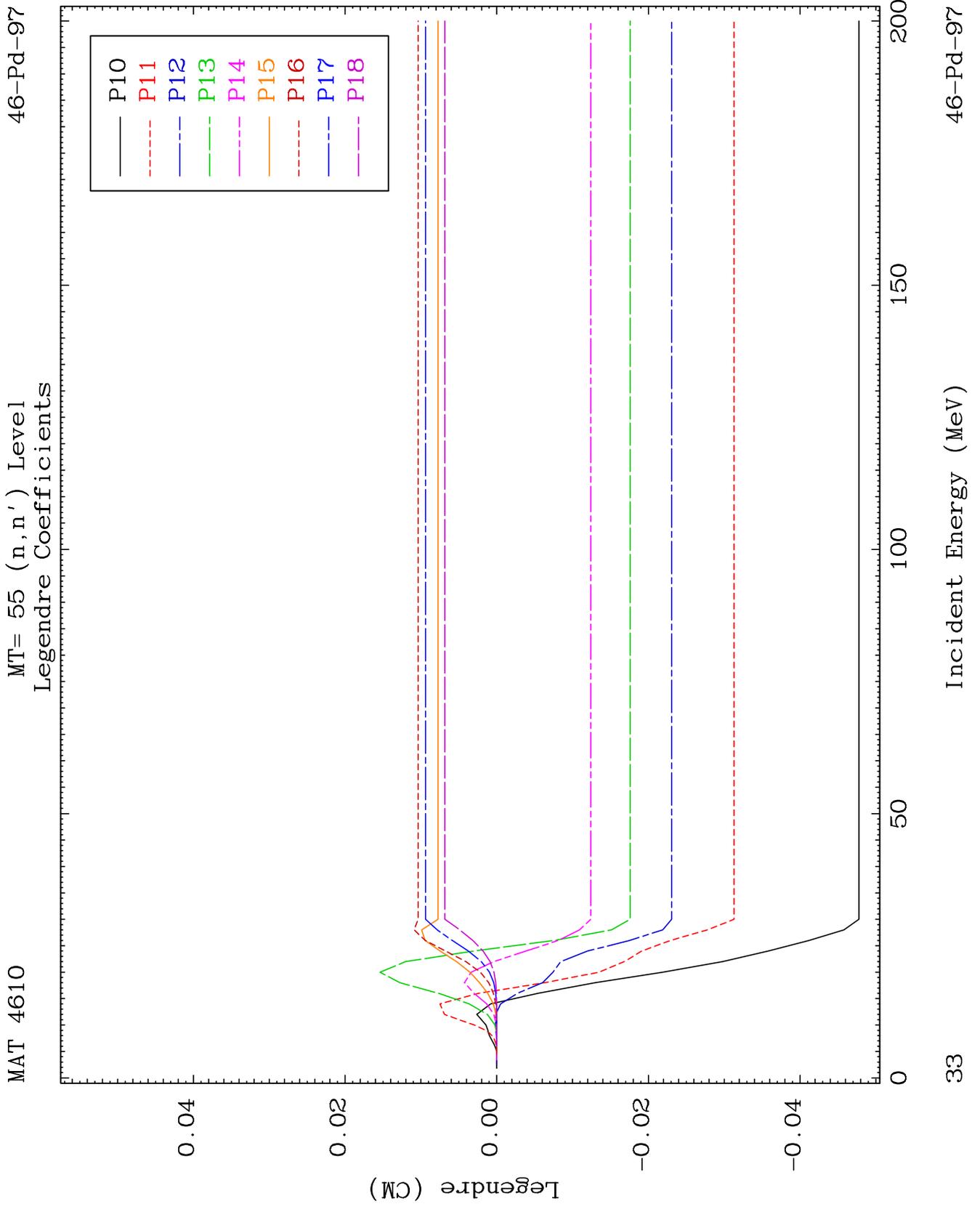
Incident Energy (MeV)

46-Pd-97





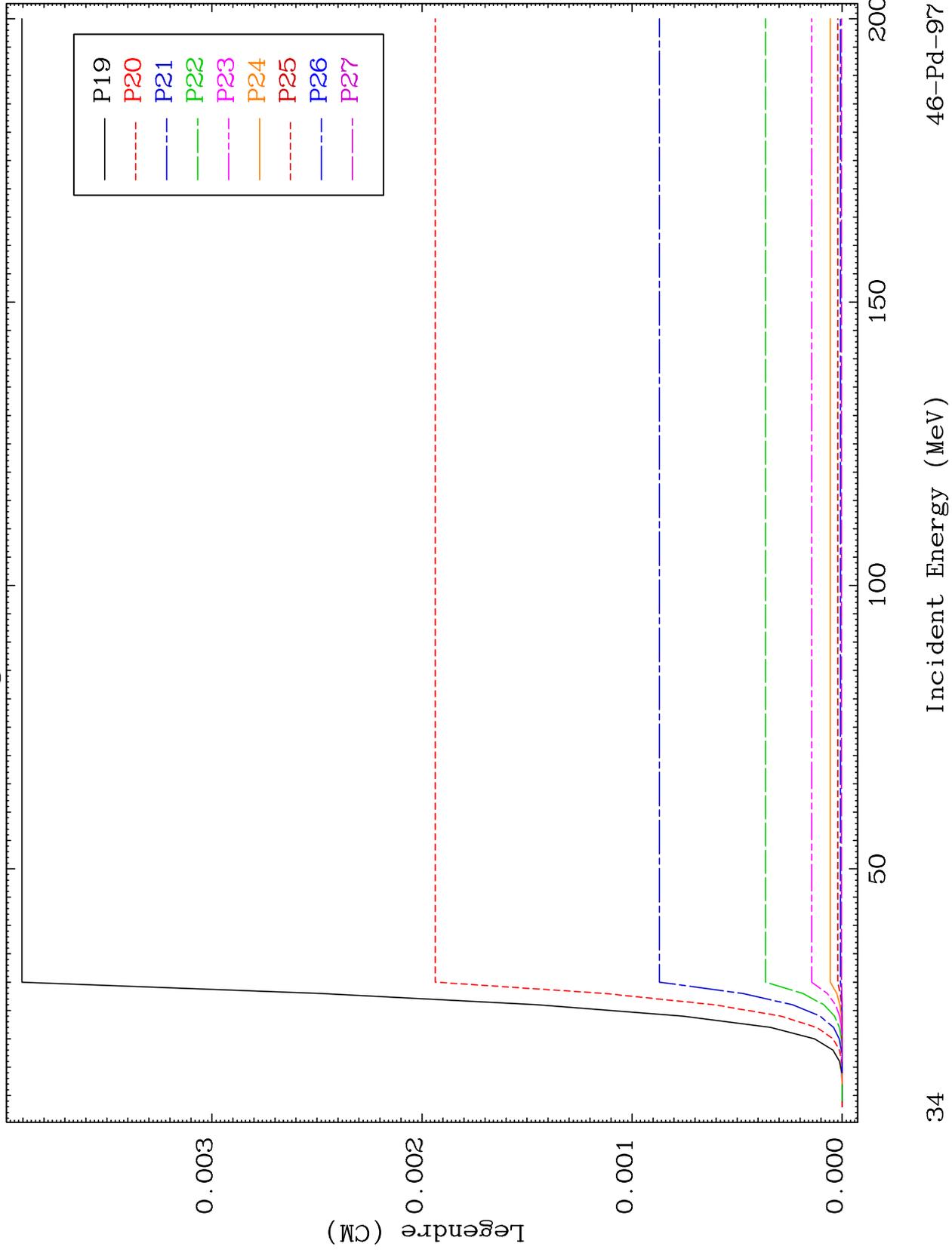




MAT 4610

MT= 55 (n,n') Level
Legendre Coefficients

46-Pd-97



34

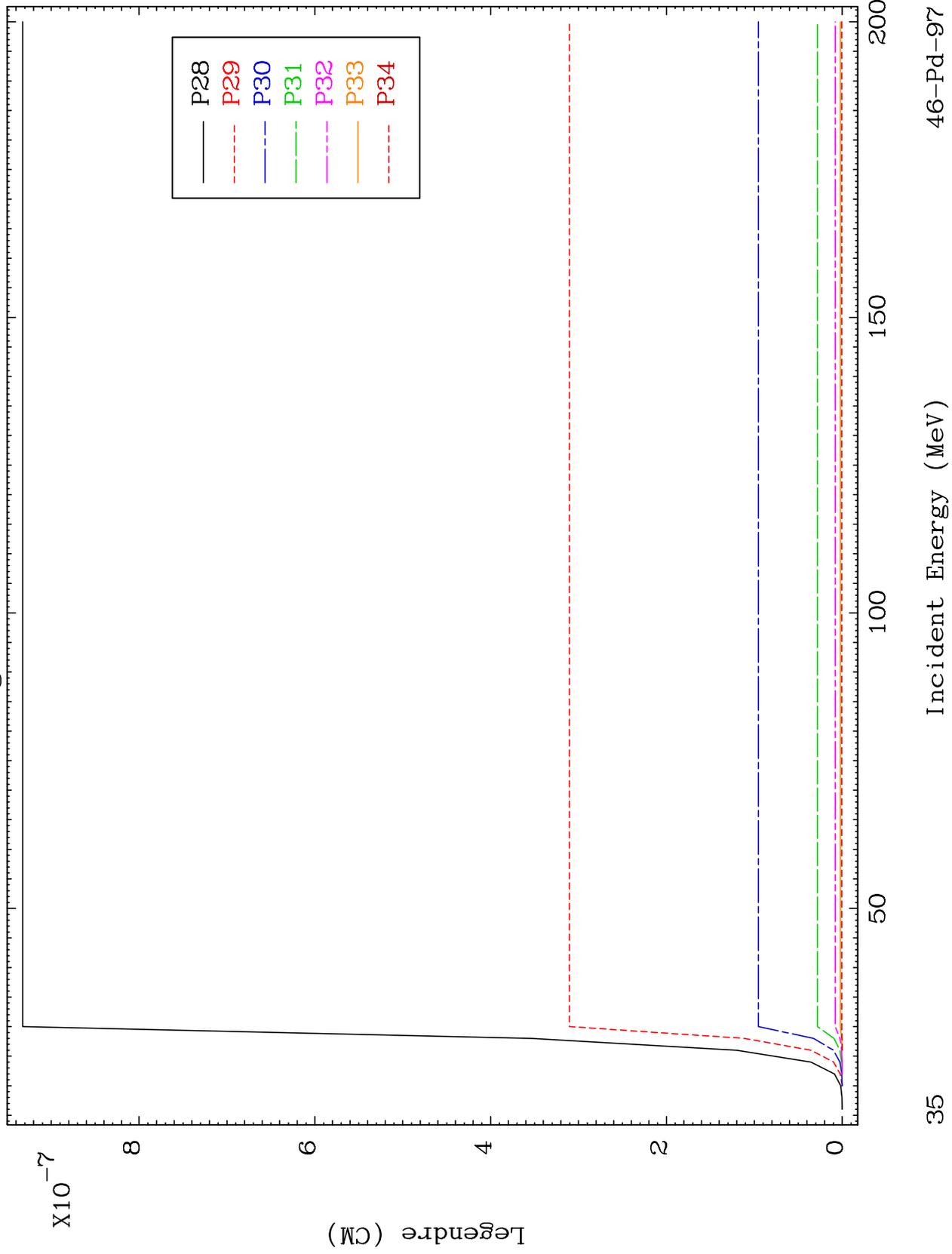
Incident Energy (MeV)

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MAT 4610

MT= 55 (n,n') Level
Legendre Coefficients

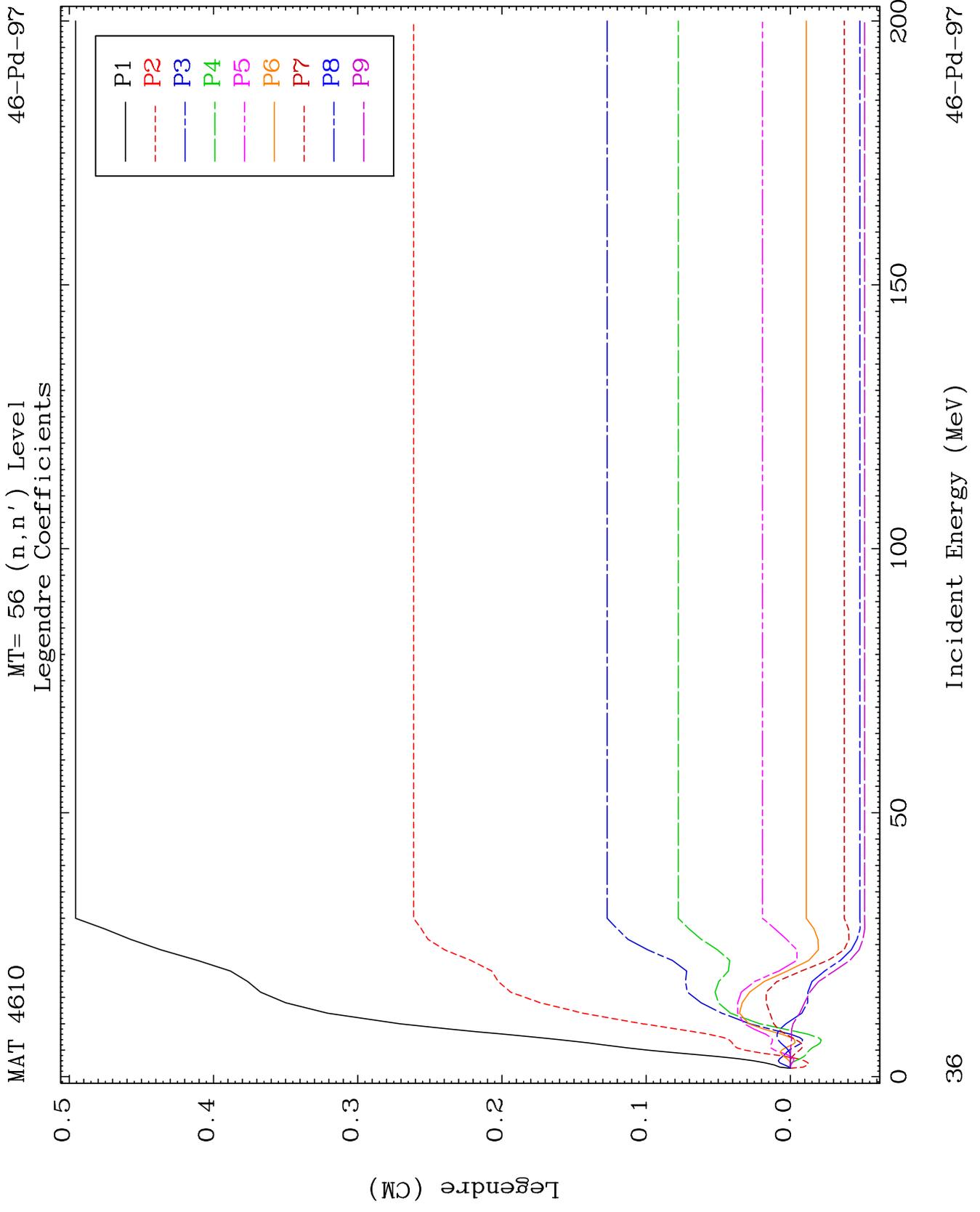
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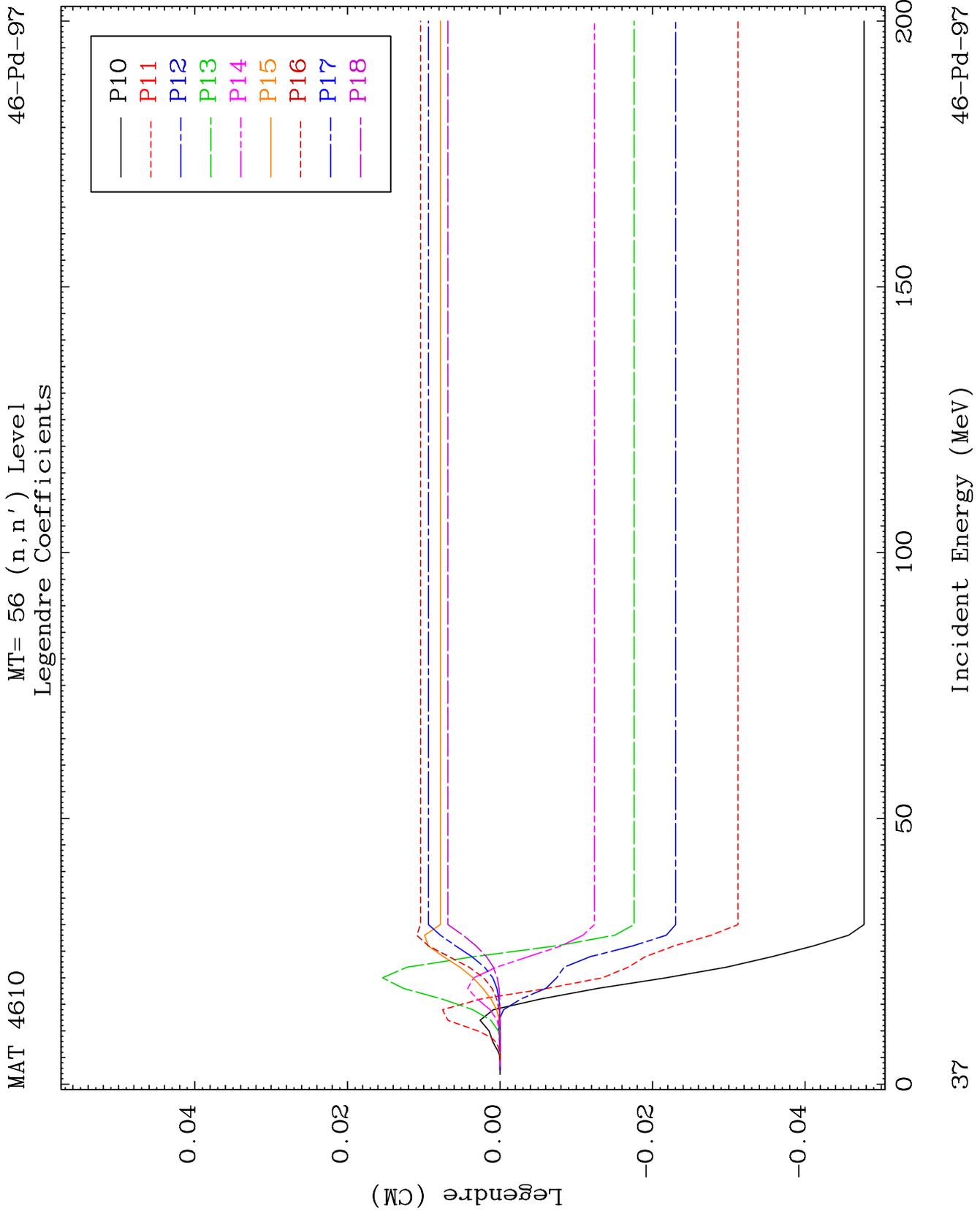


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Incident Energy (MeV)

46-Pd-97

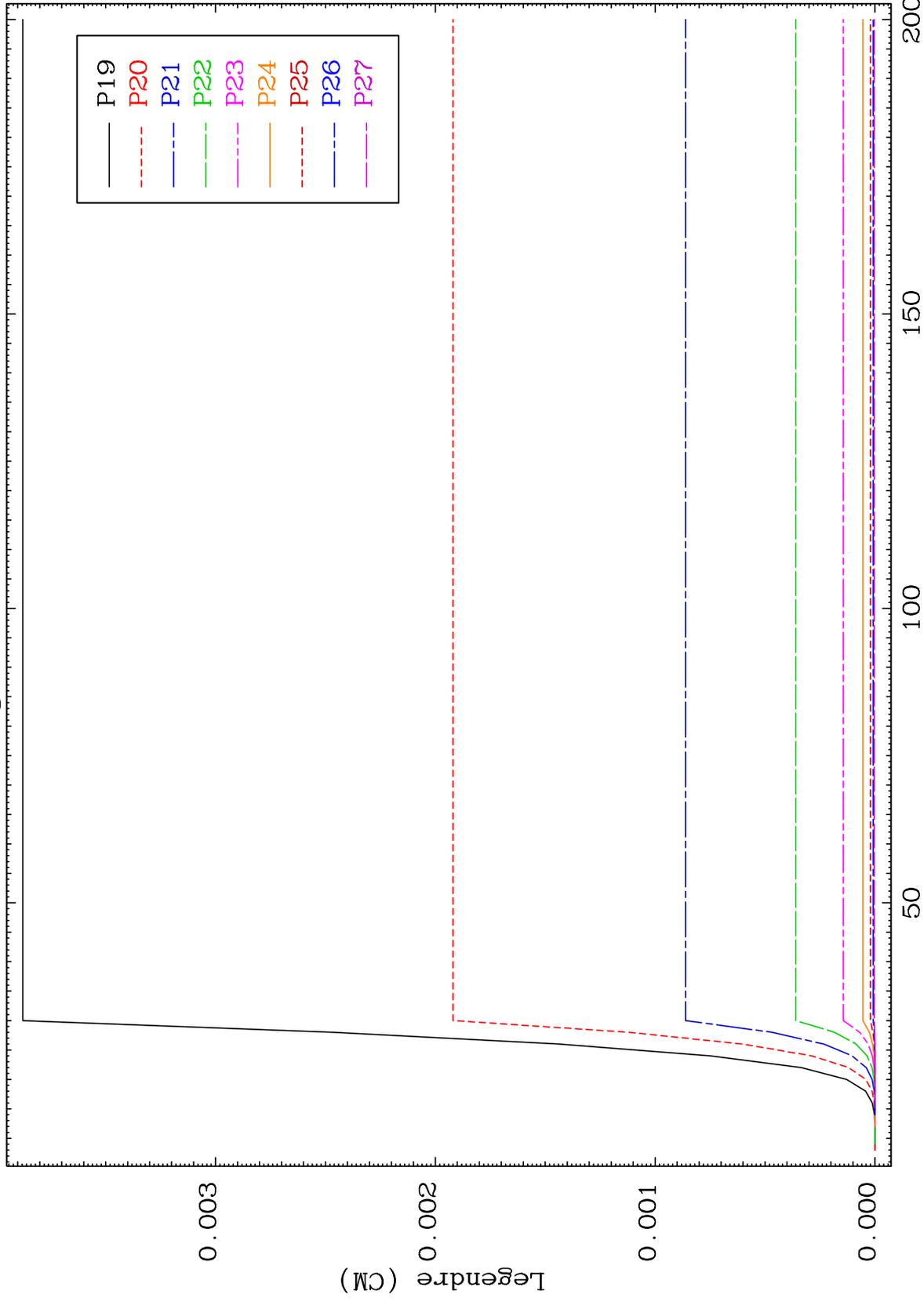




MAT 4610

MT= 56 (n,n') Level
Legendre Coefficients

46-Pd-97



38

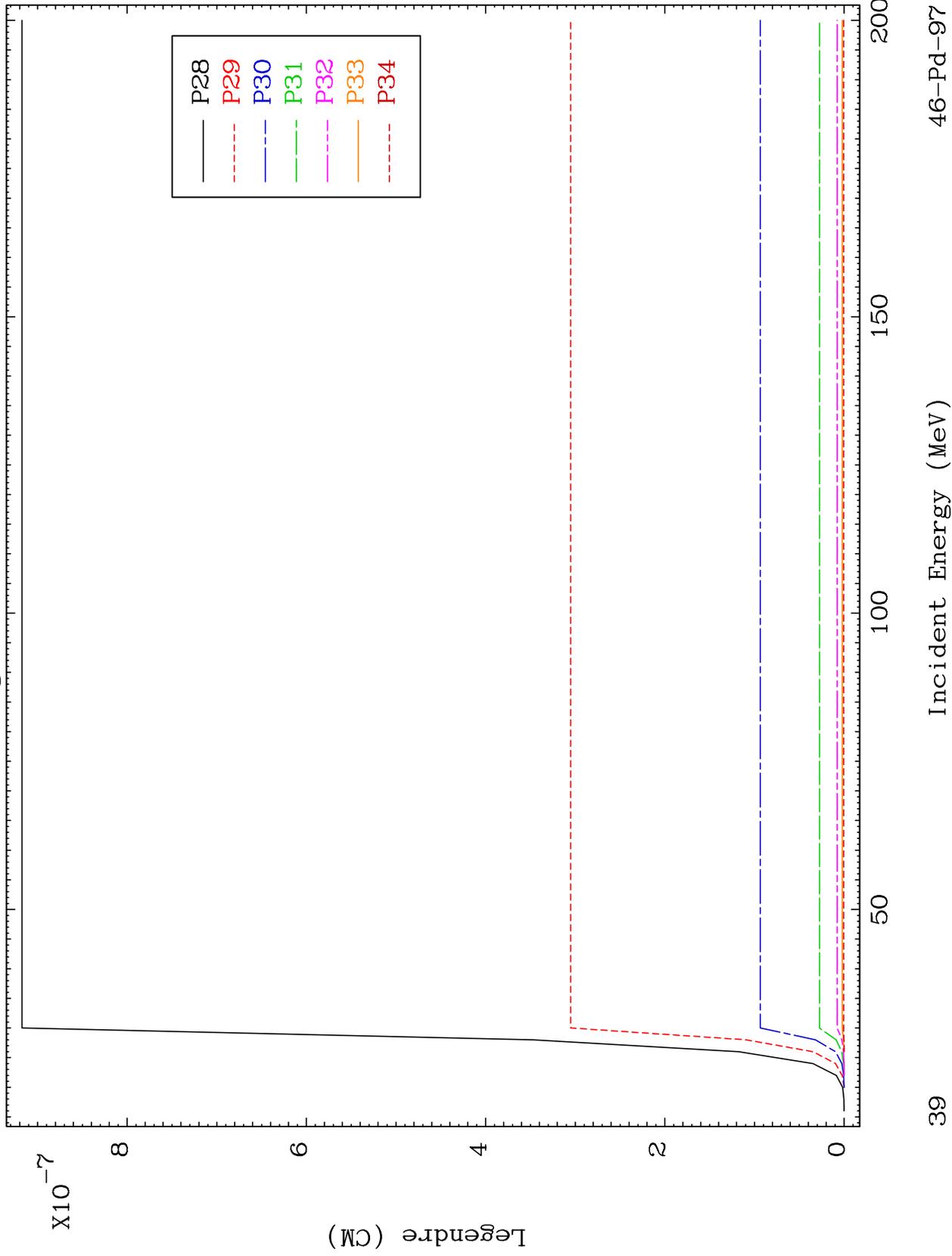
Incident Energy (MeV)

46-Pd-97

MAT 4610

MT= 56 (n,n') Level
Legendre Coefficients

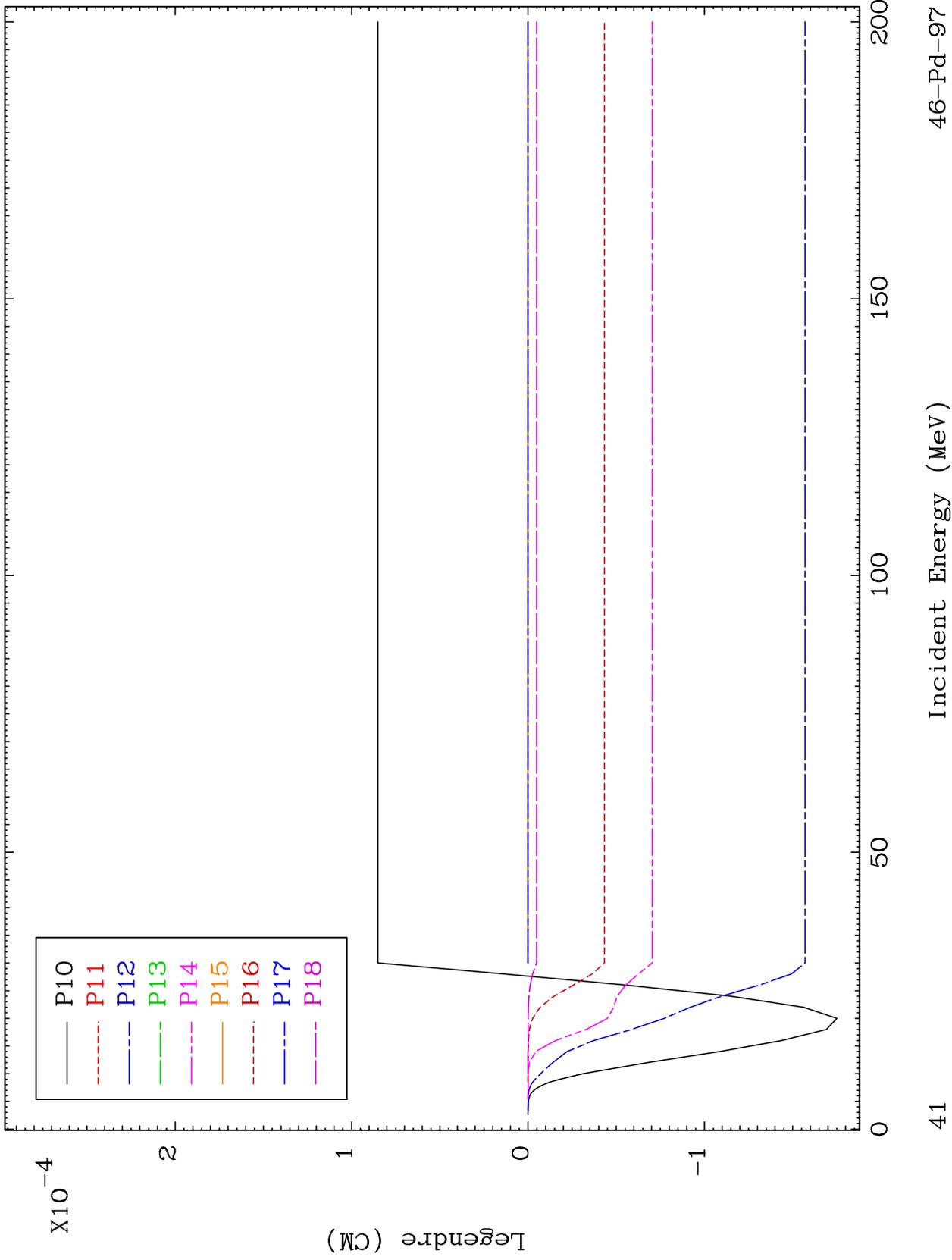
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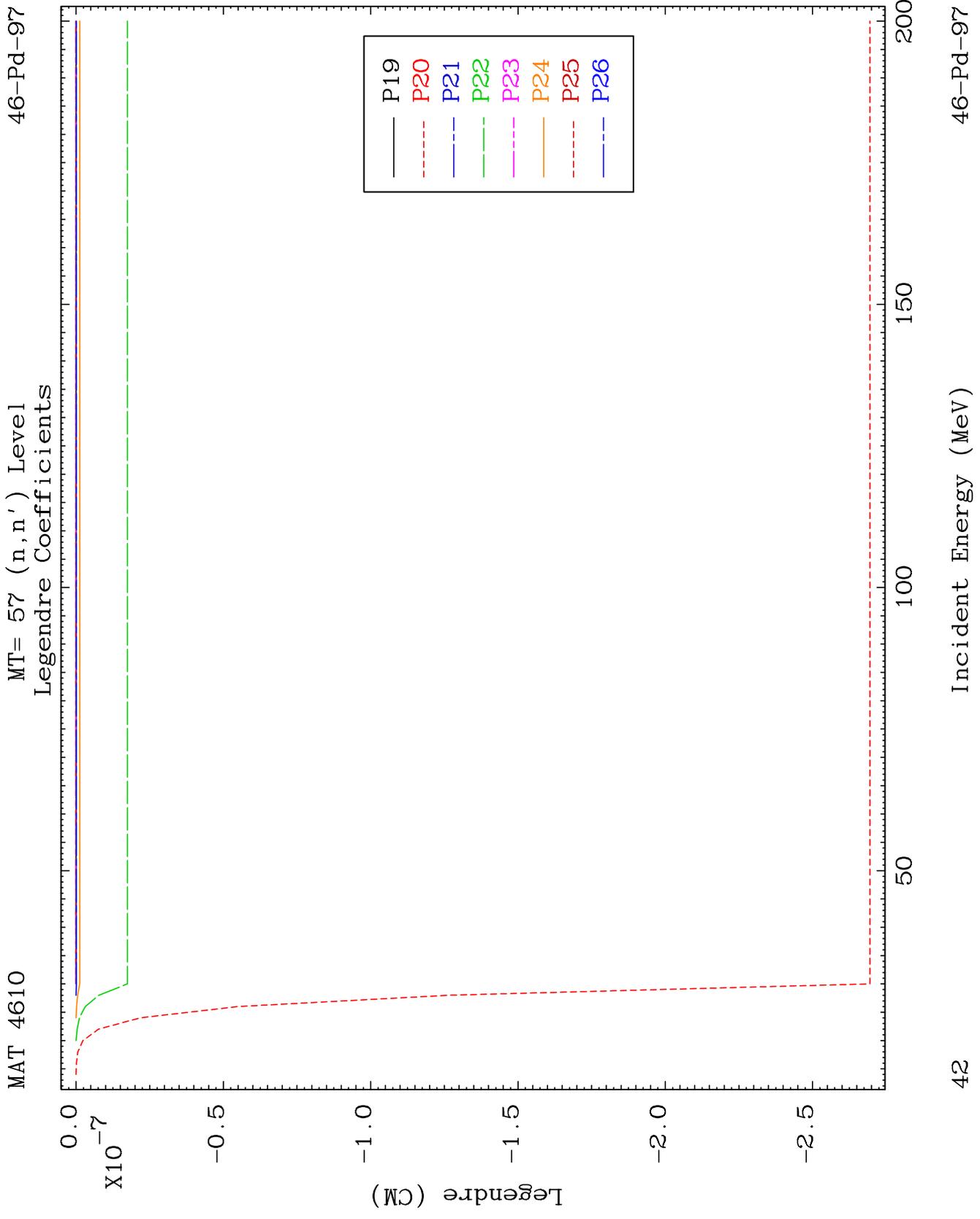


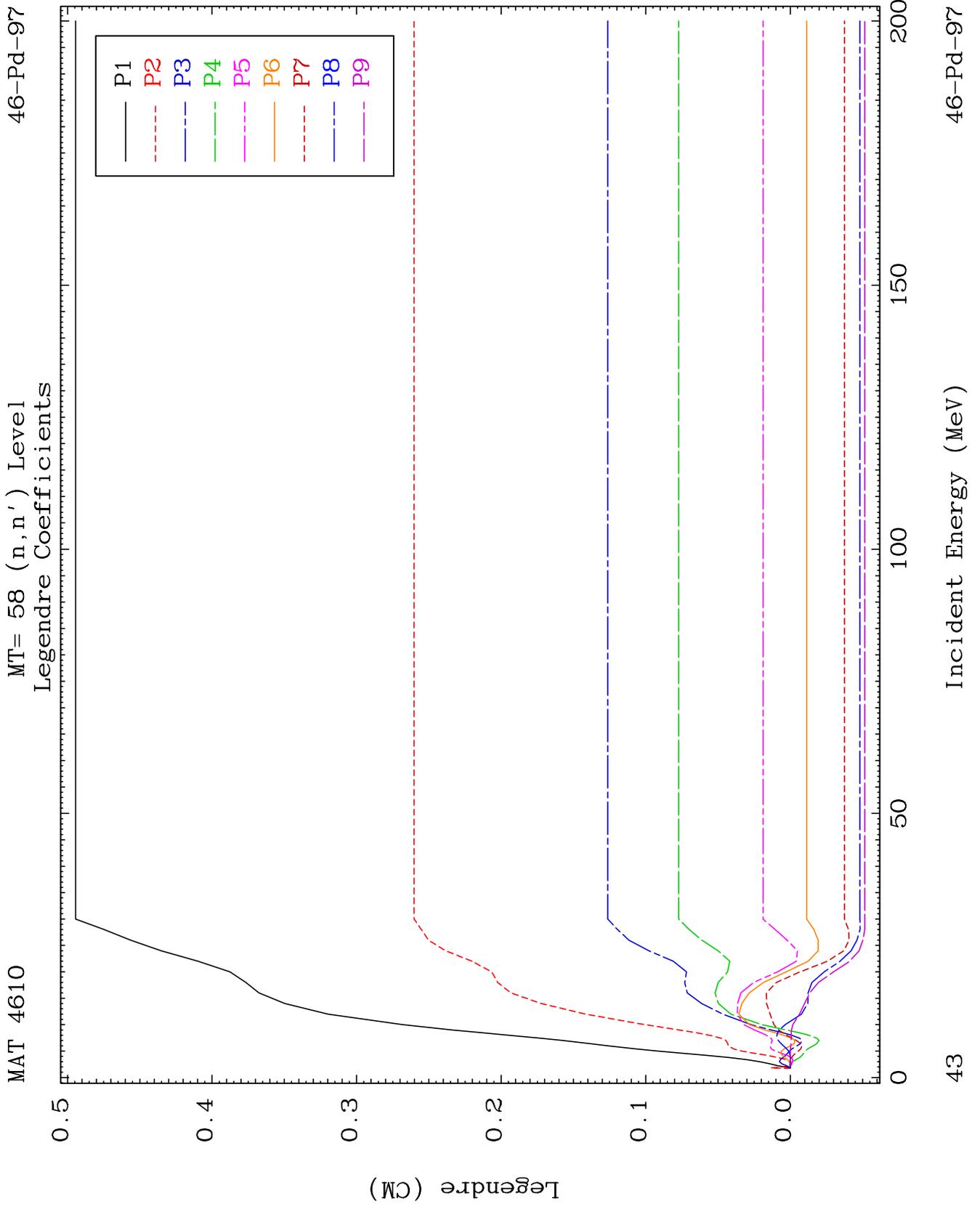
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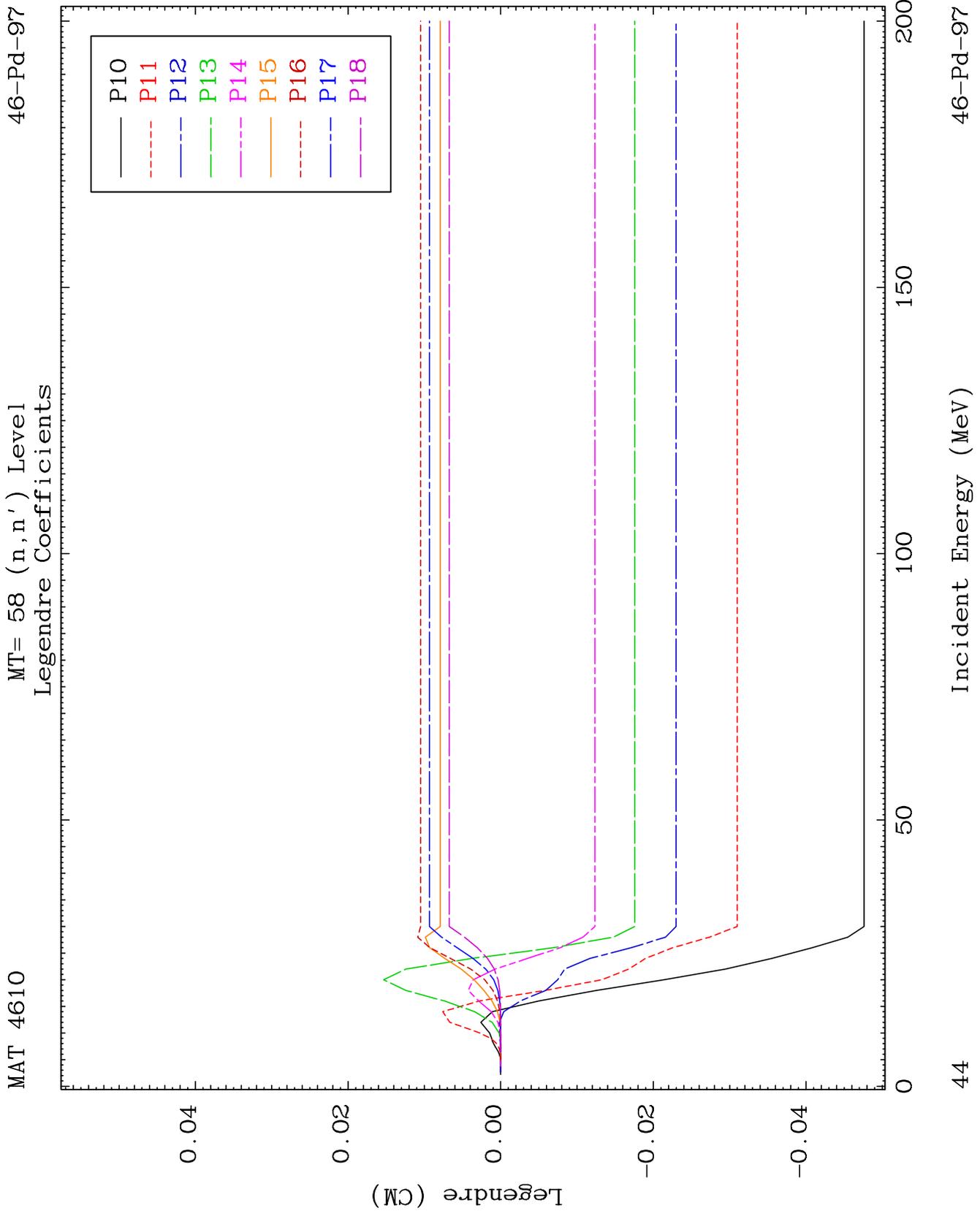
MT= 57 (n,n') Level
Legendre Coefficients

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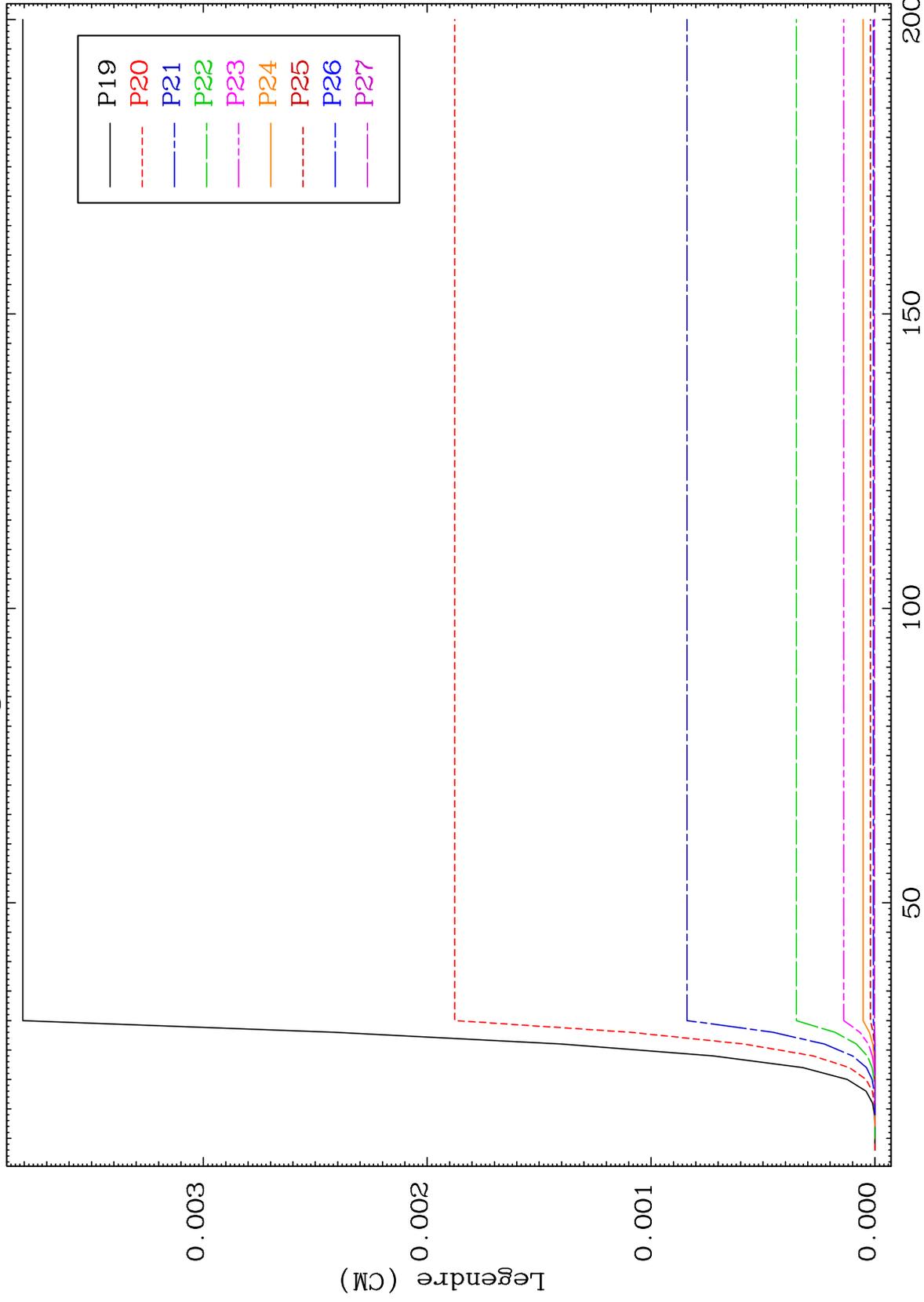




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MT= 58 (n,n') Level
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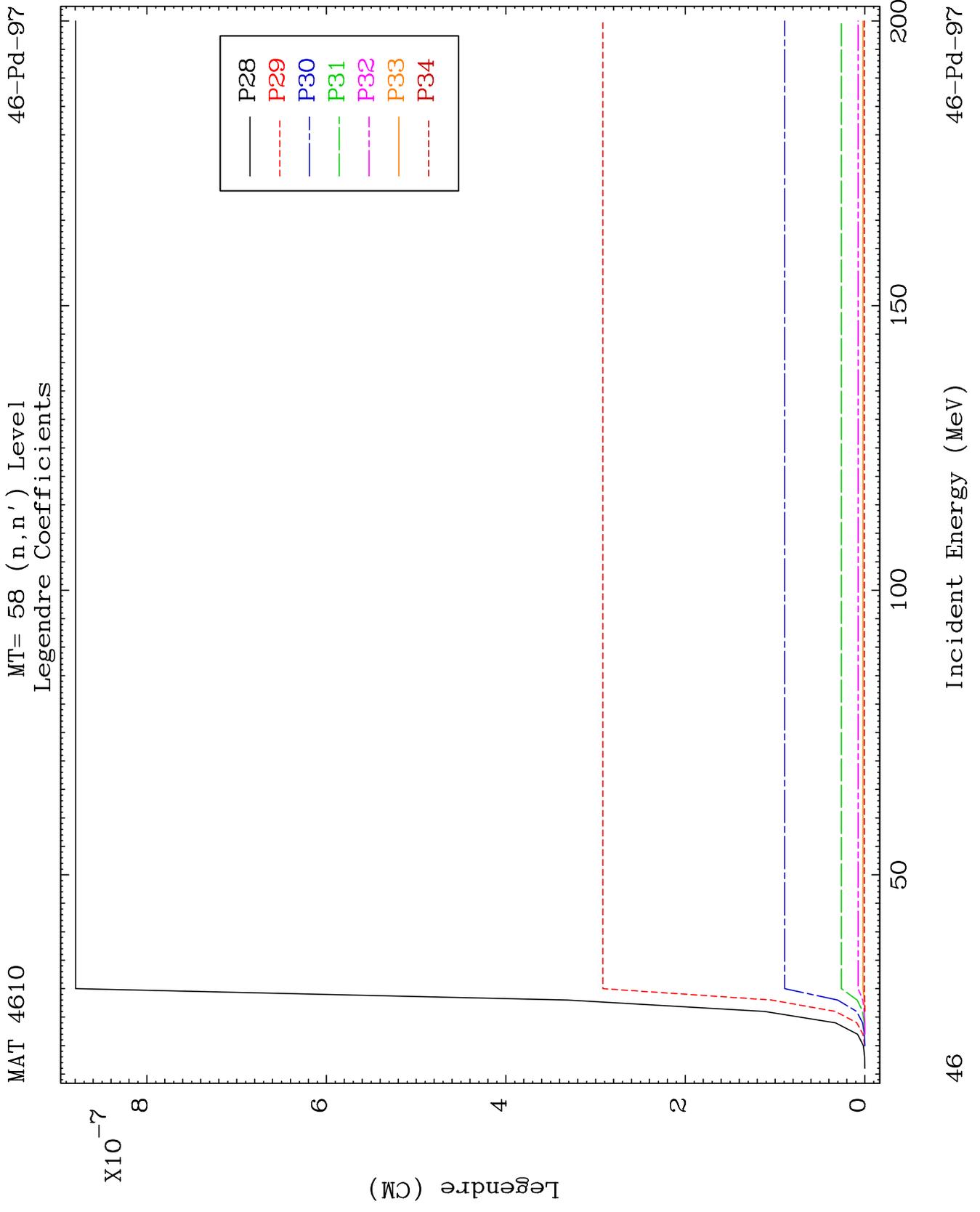
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Incident Energy (MeV)

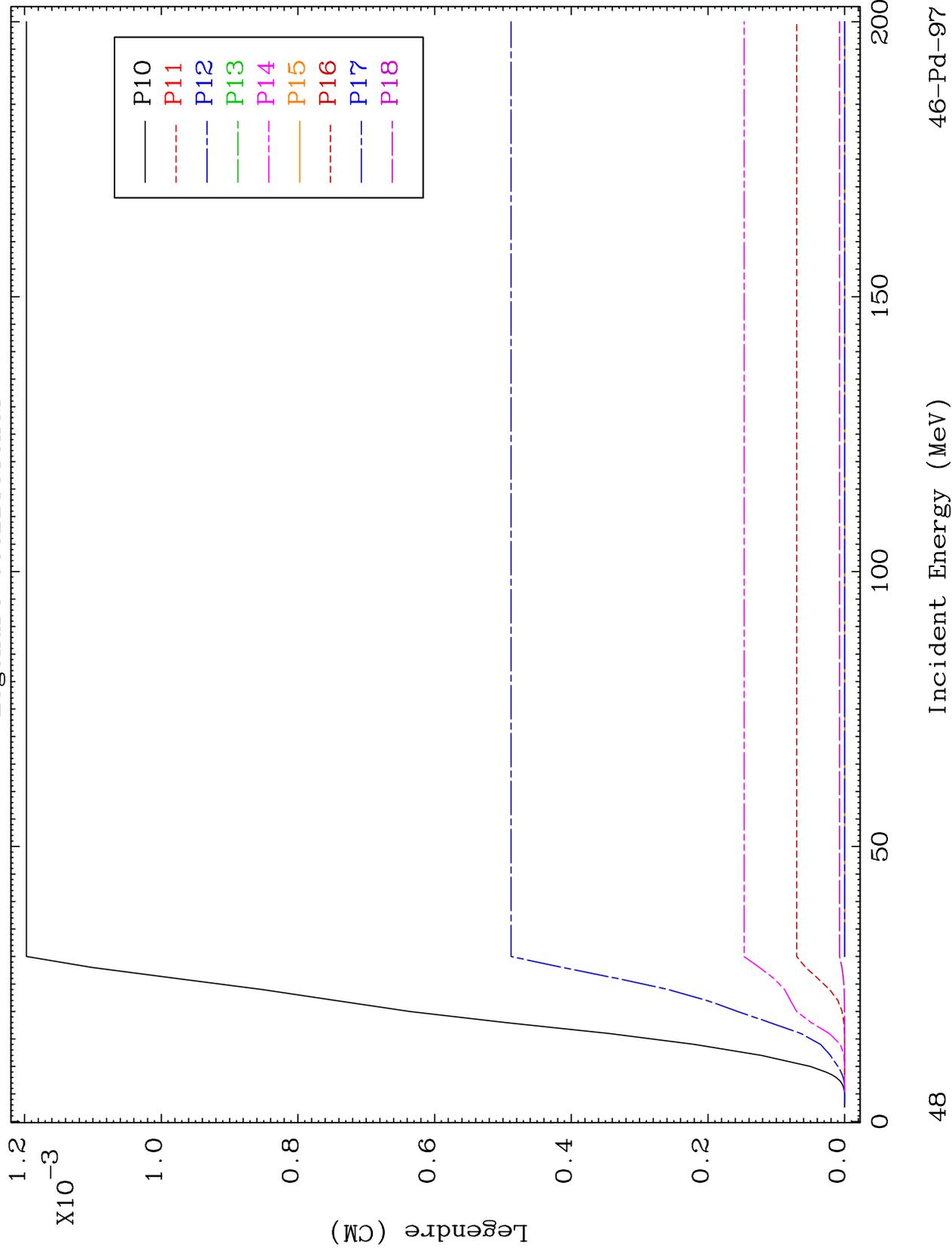
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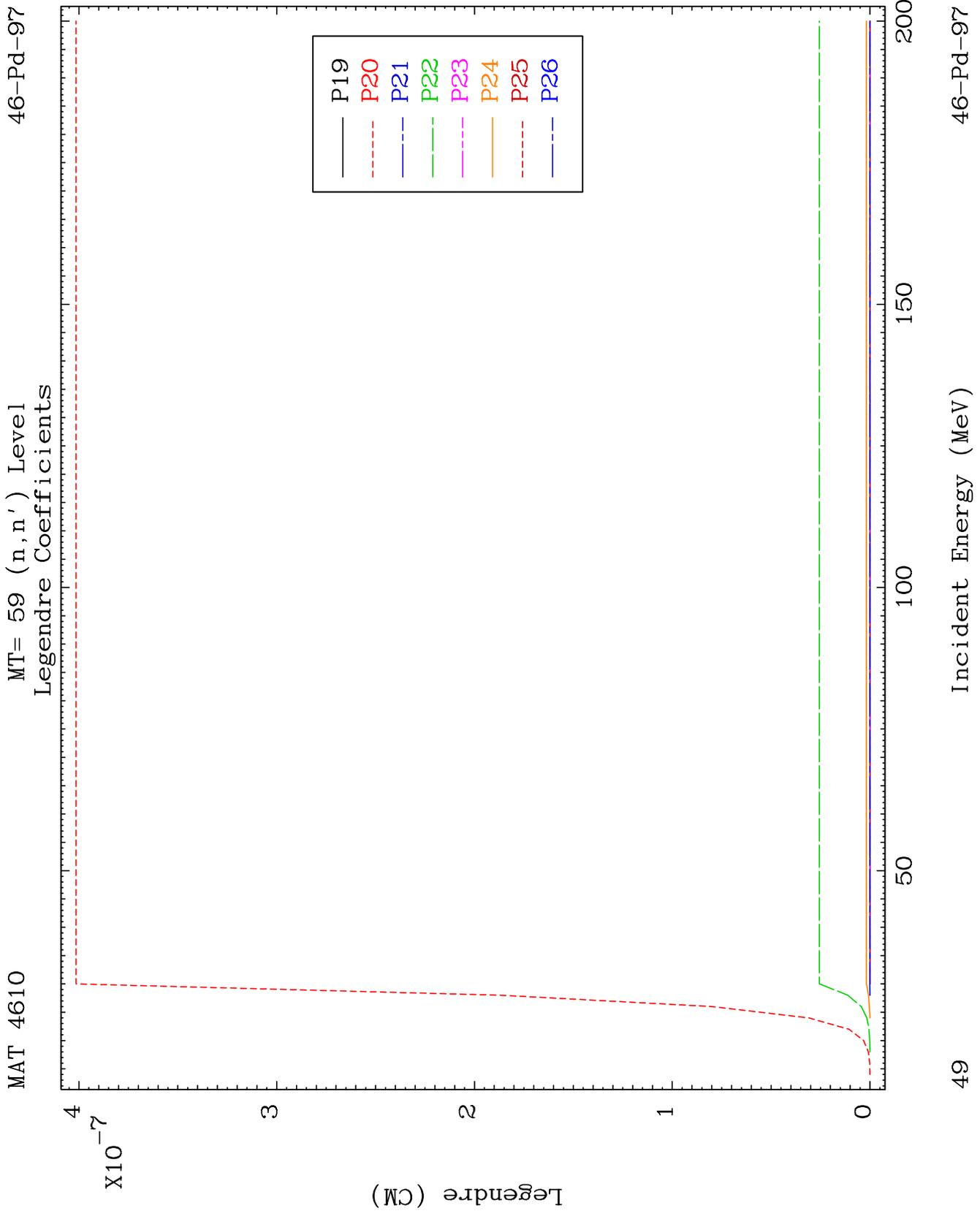


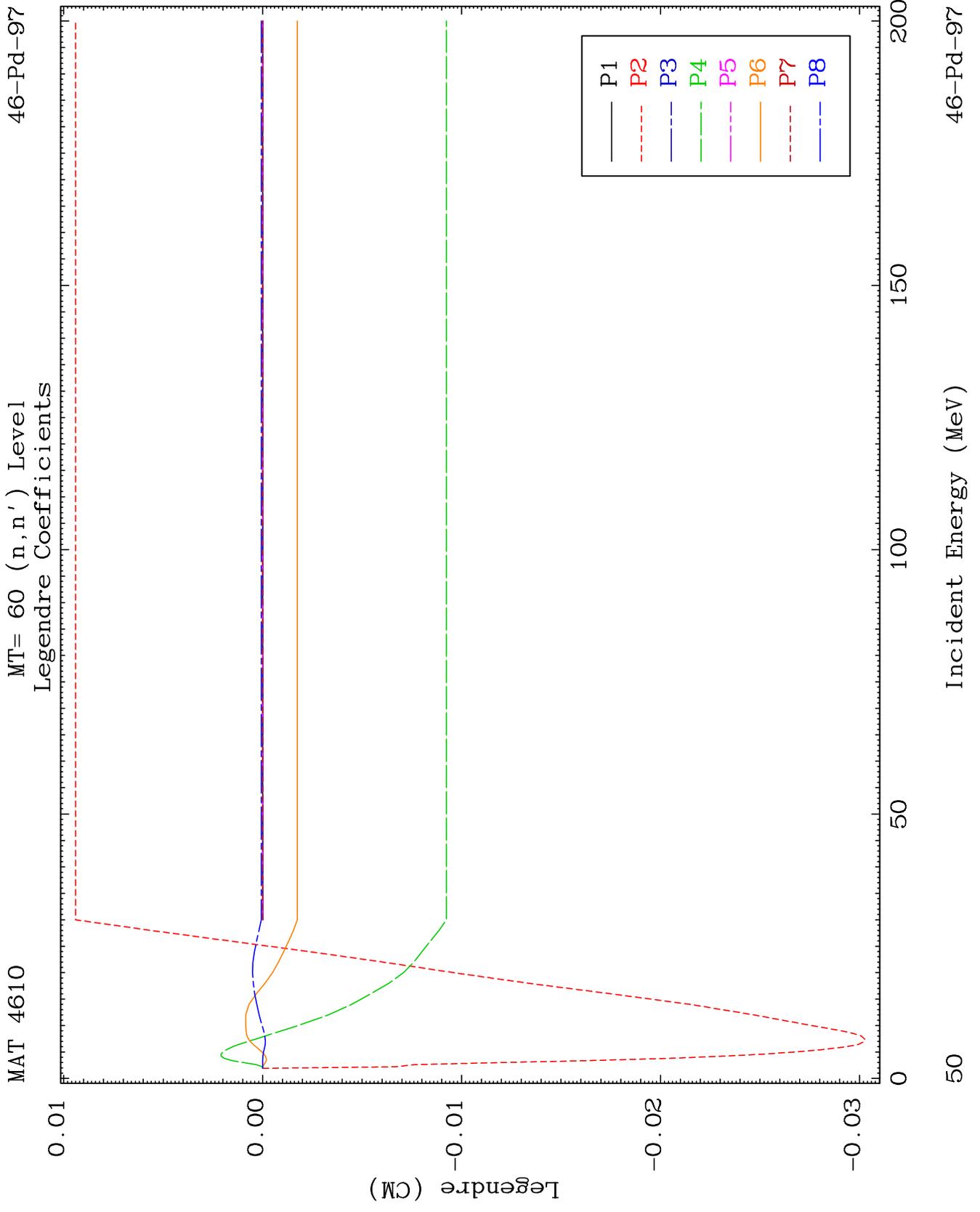
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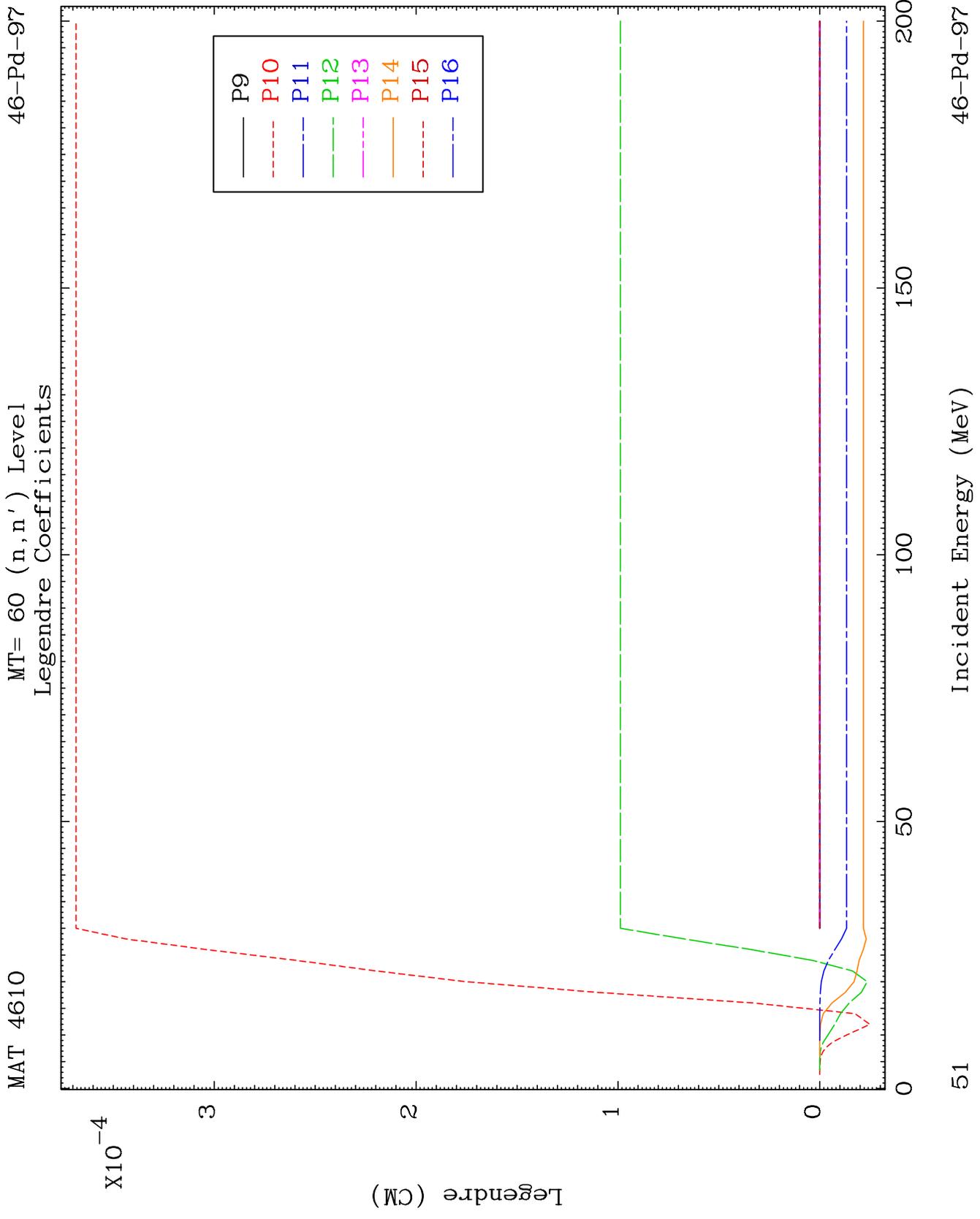
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Legendre Coefficients

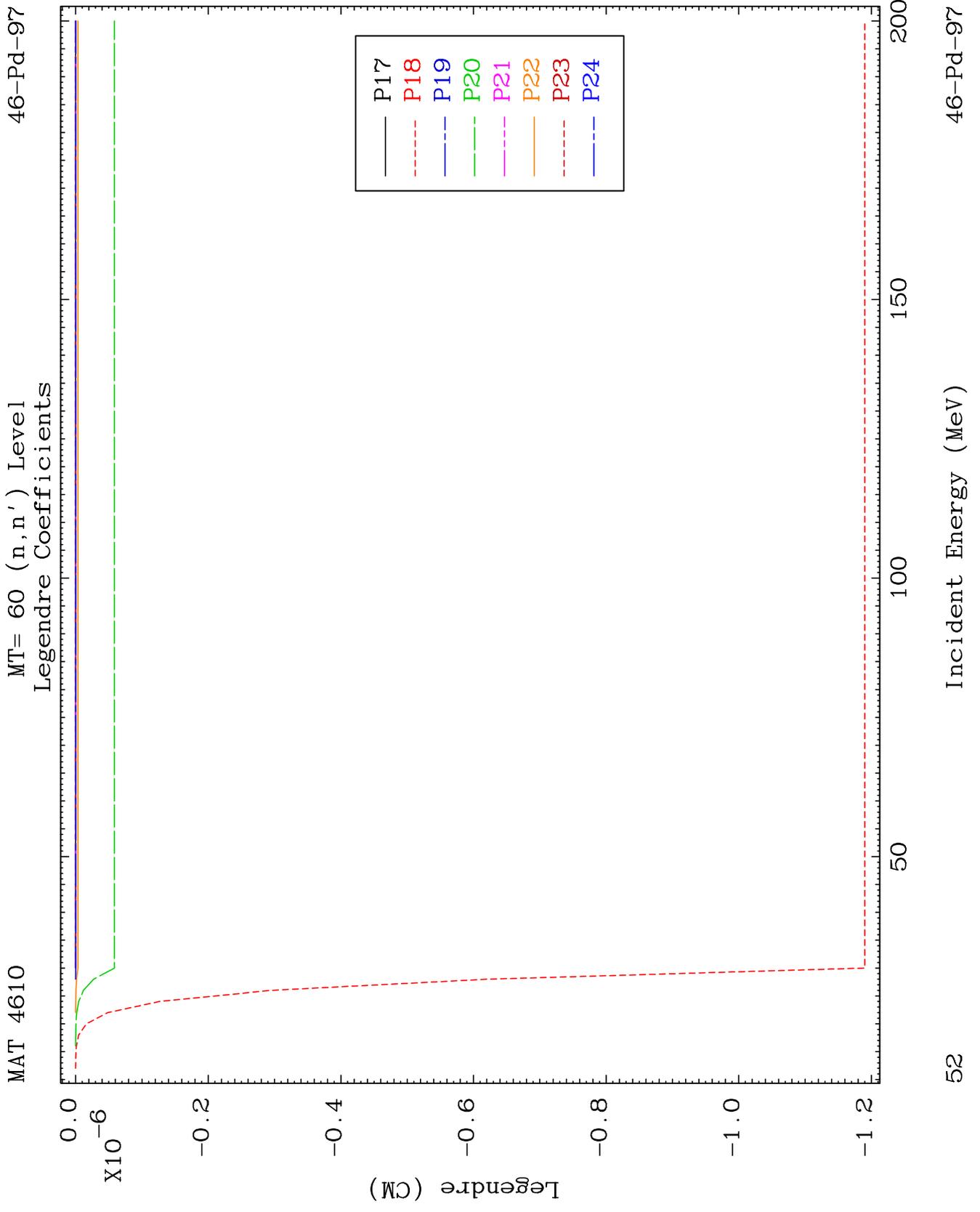
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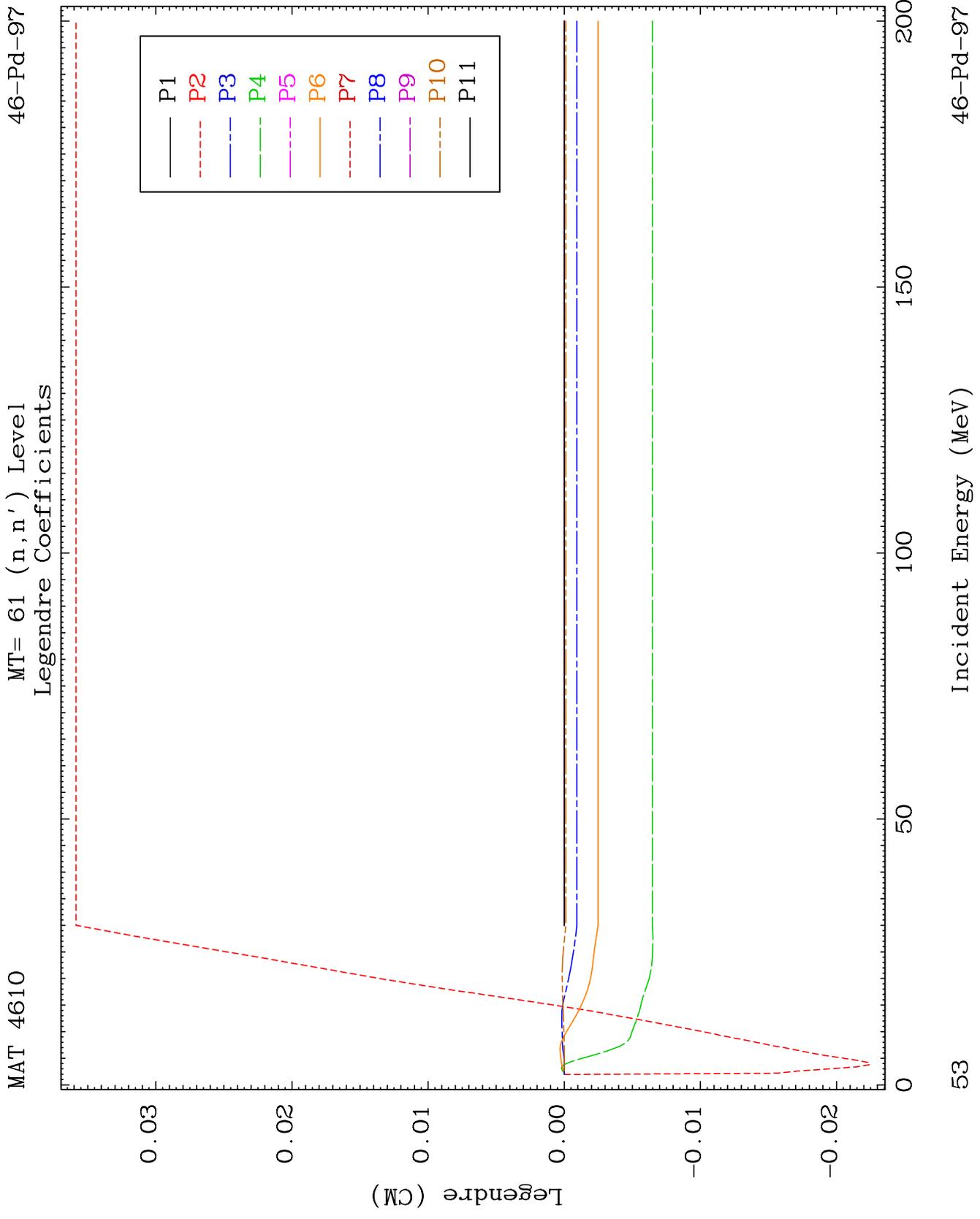








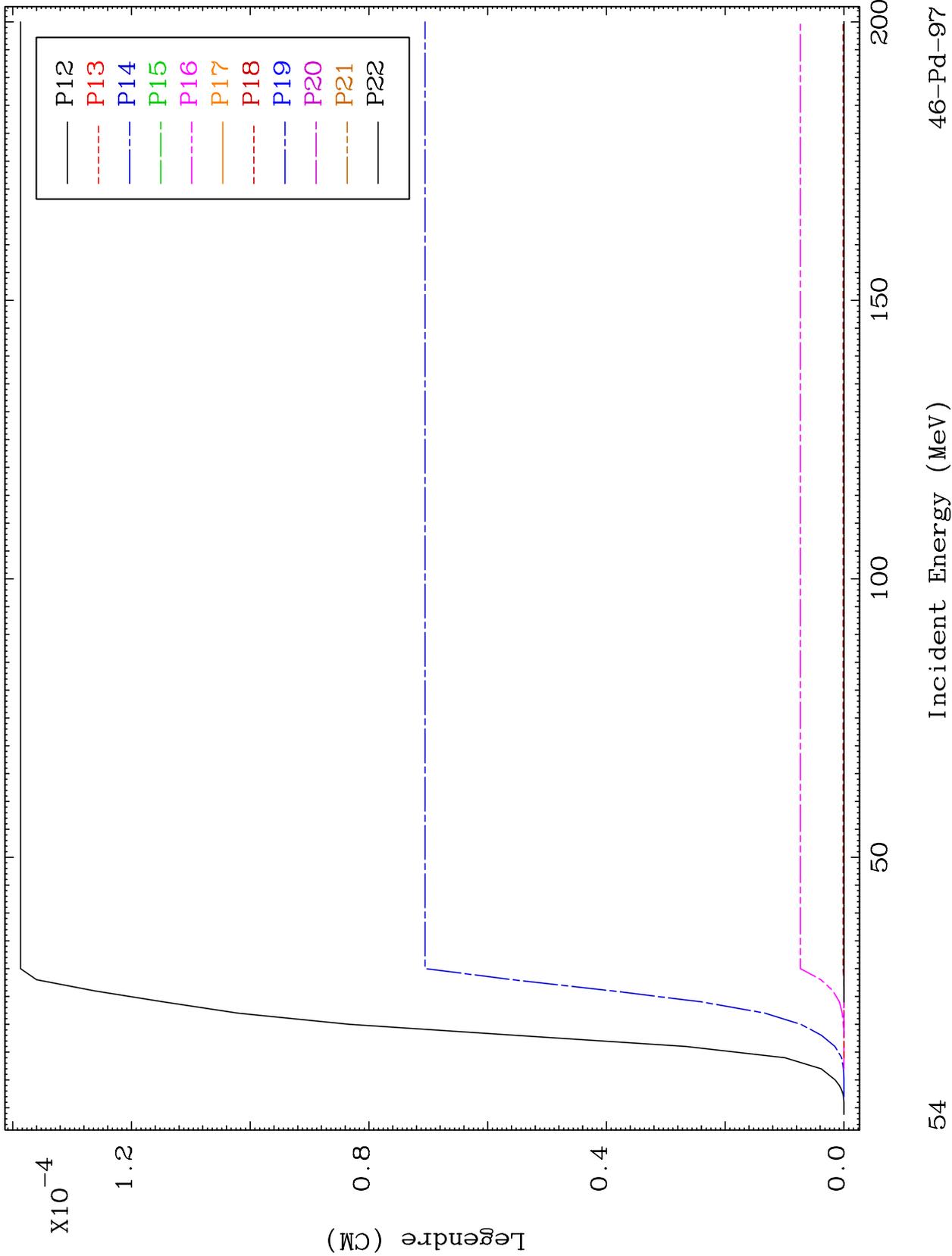




MAT 4610

MT= 61 (n,n') Level
Legendre Coefficients

46-Pd-97



54

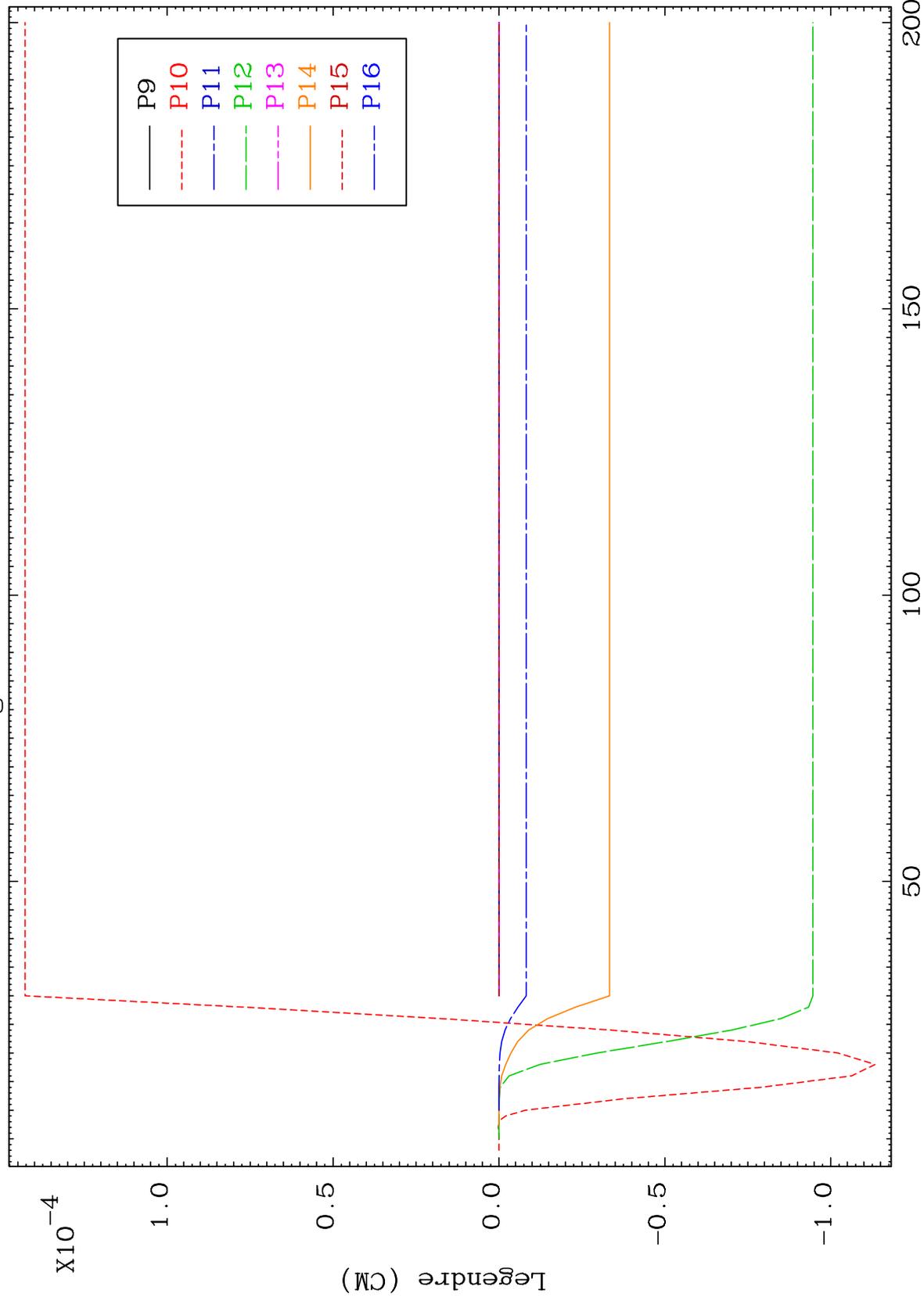
Incident Energy (MeV)

46-Pd-97

MAT 4610

MT= 62 (n,n') Level
Legendre Coefficients

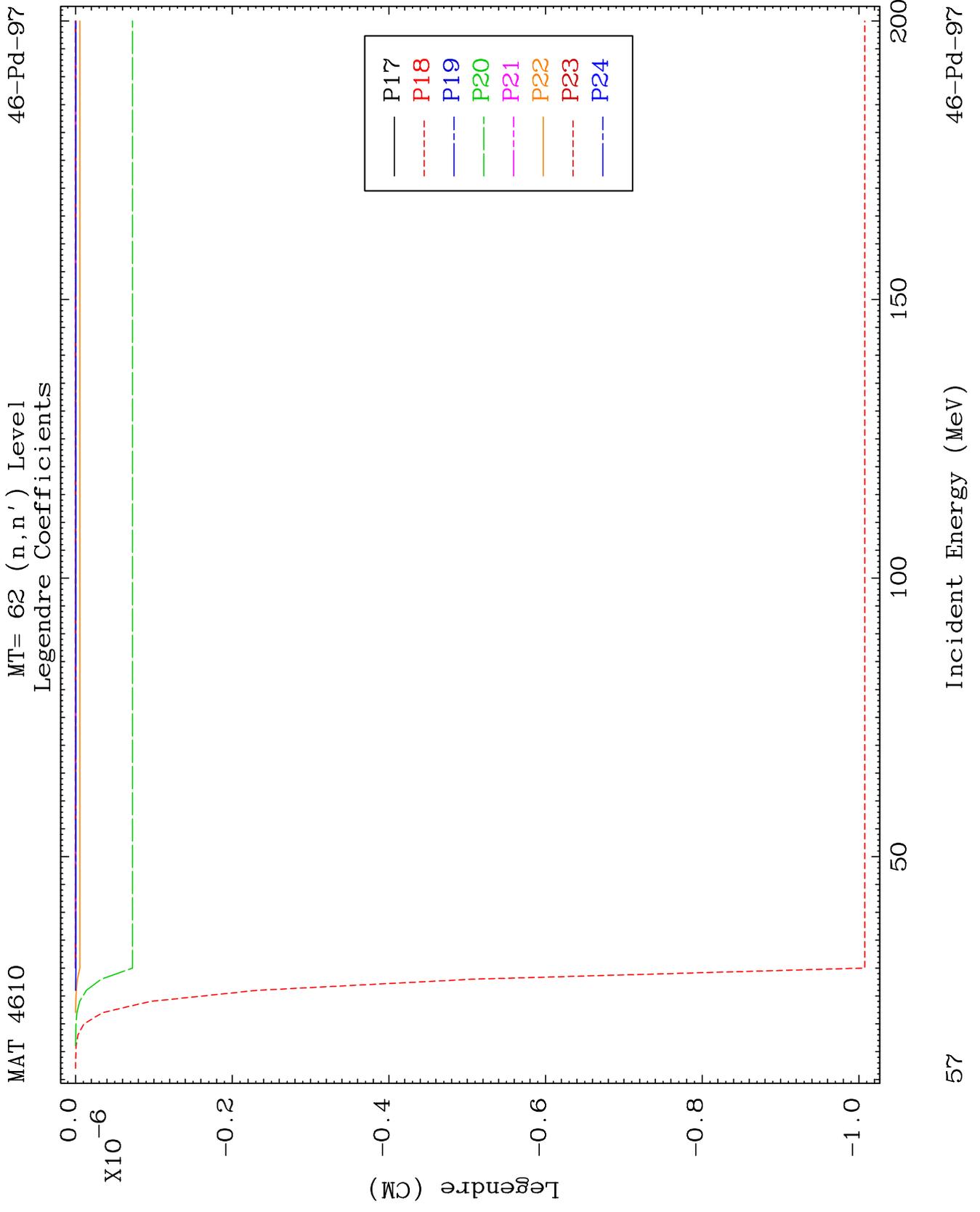
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56

Incident Energy (MeV)

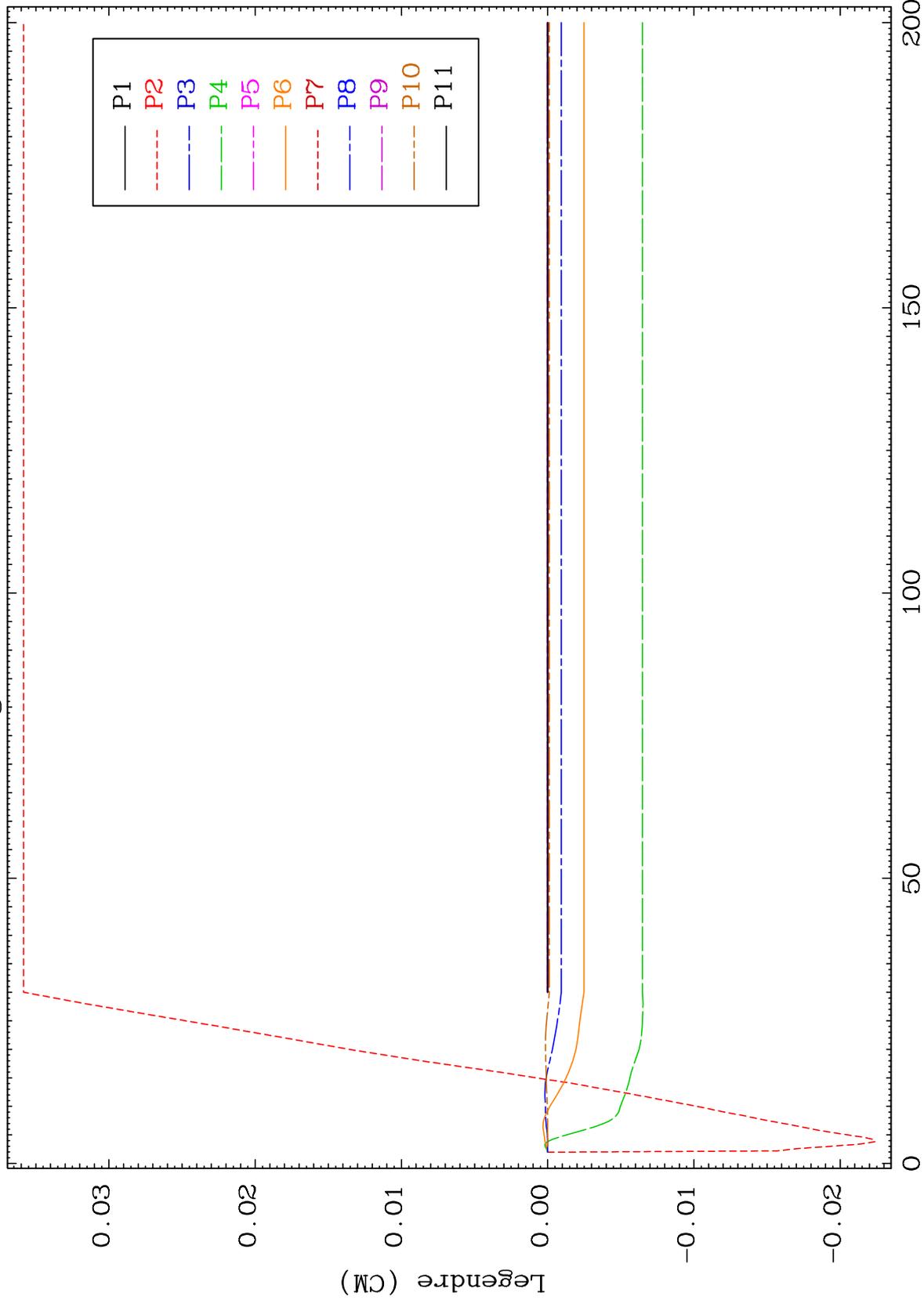
46-Pd-97



MAT 4610

MT= 63 (n,n') Level
Legendre Coefficients

46-Pd-97



46-Pd-97

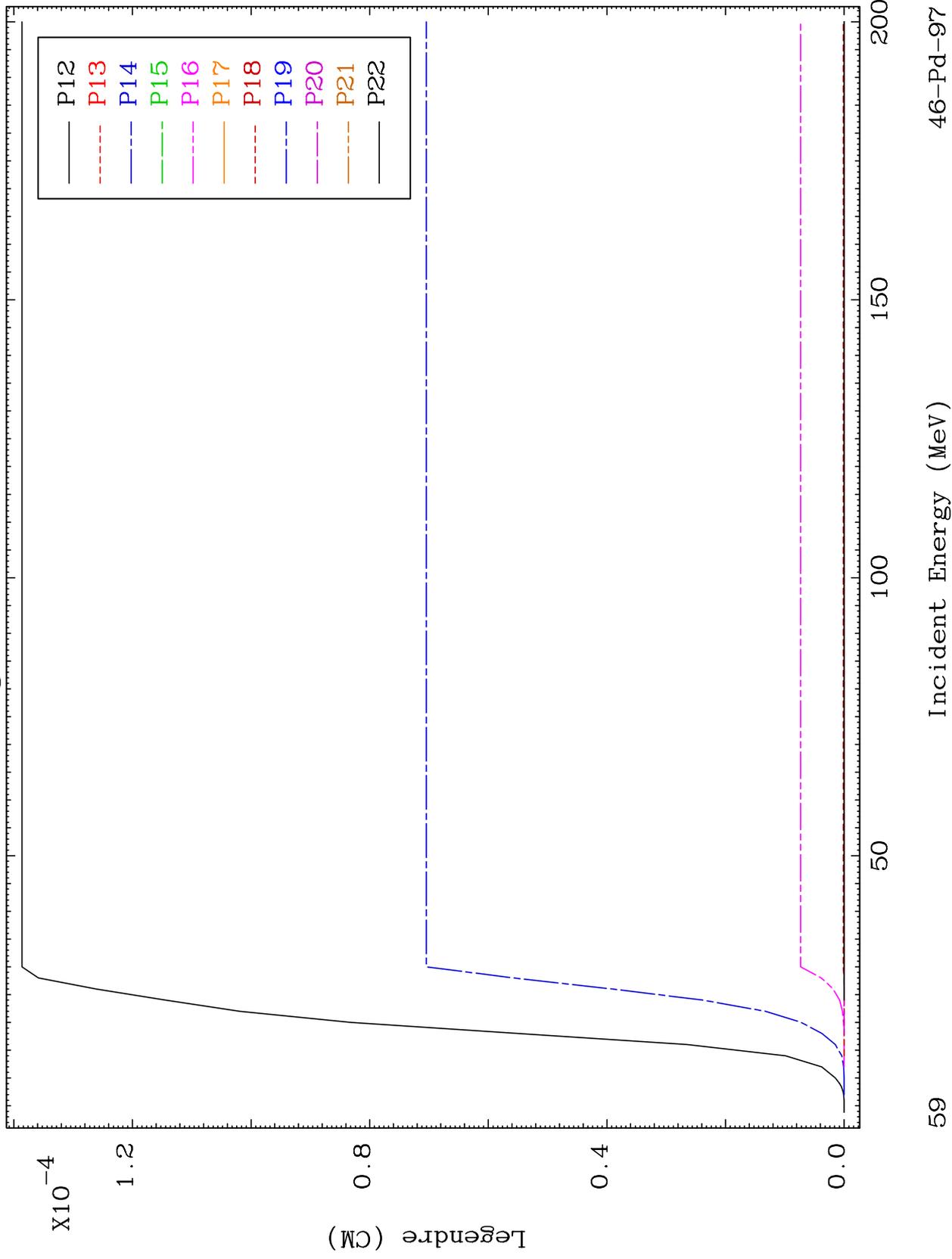
Incident Energy (MeV)

58

MAT 4610

MT= 63 (n,n') Level
Legendre Coefficients

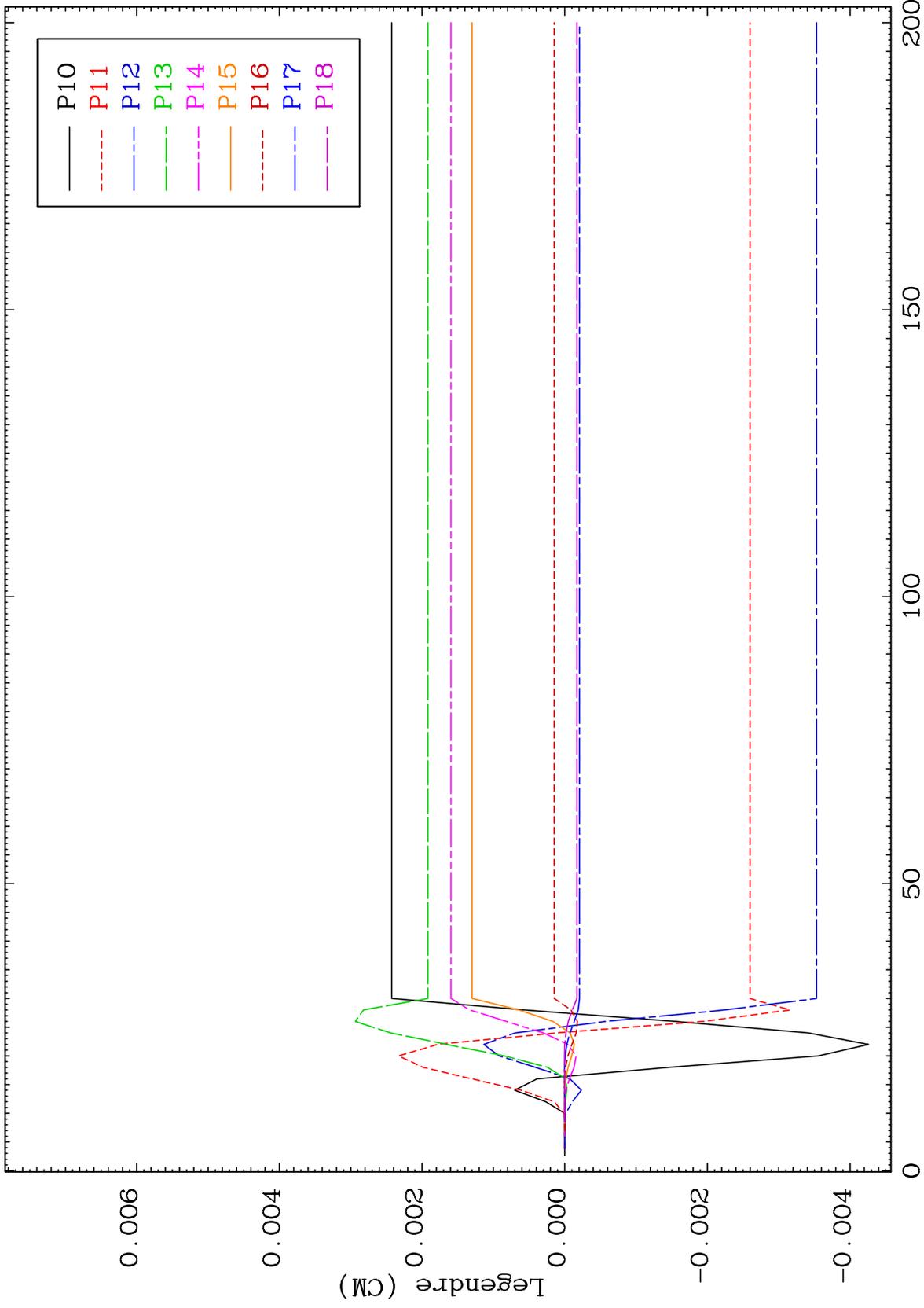
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MAT 4610

MT= 64 (n,n') Level
Legendre Coefficients

46-Pd-97



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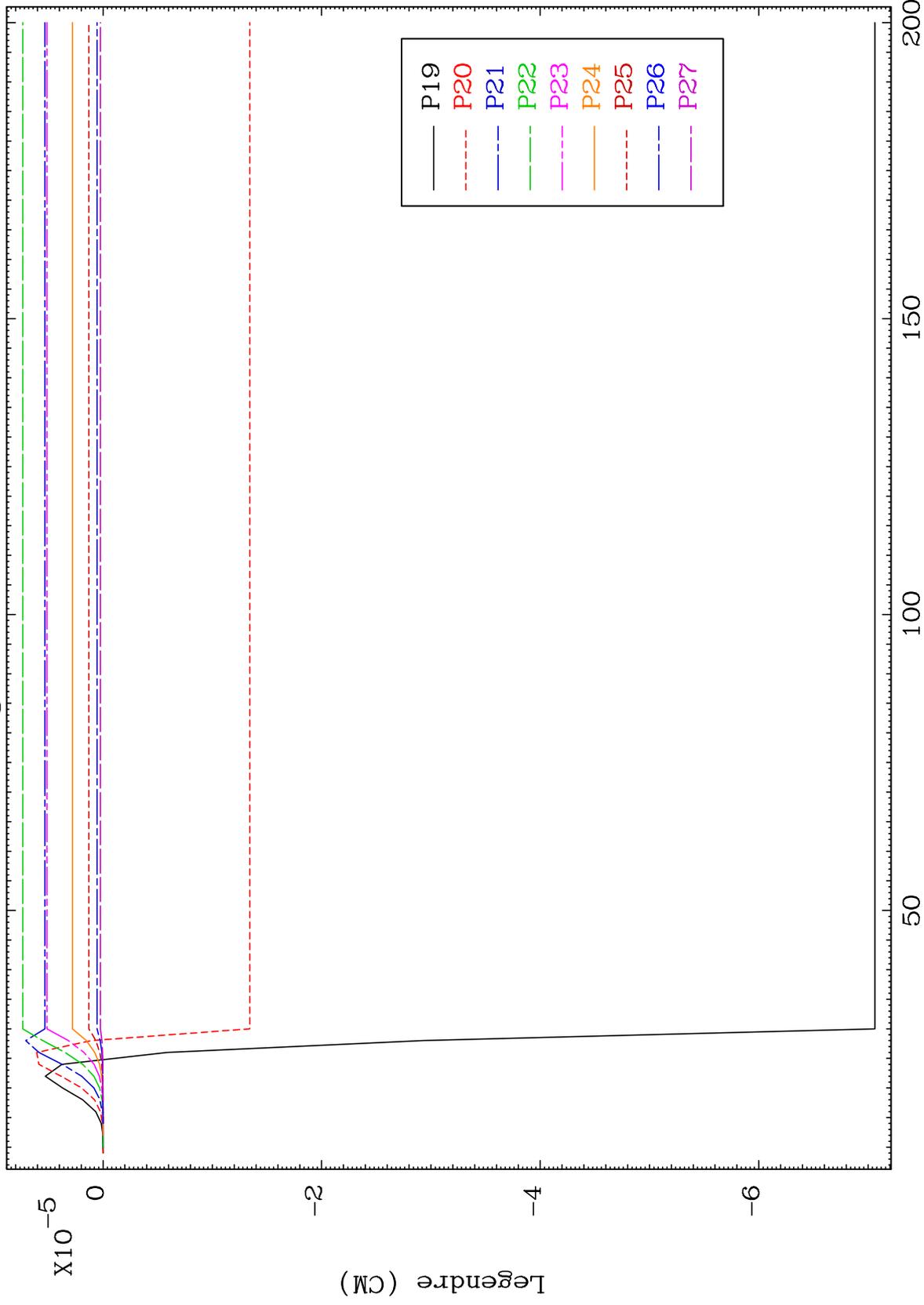
Incident Energy (MeV)

46-Pd-97

MAT 4610

MT= 64 (n,n') Level
Legendre Coefficients

46-Pd-97



62

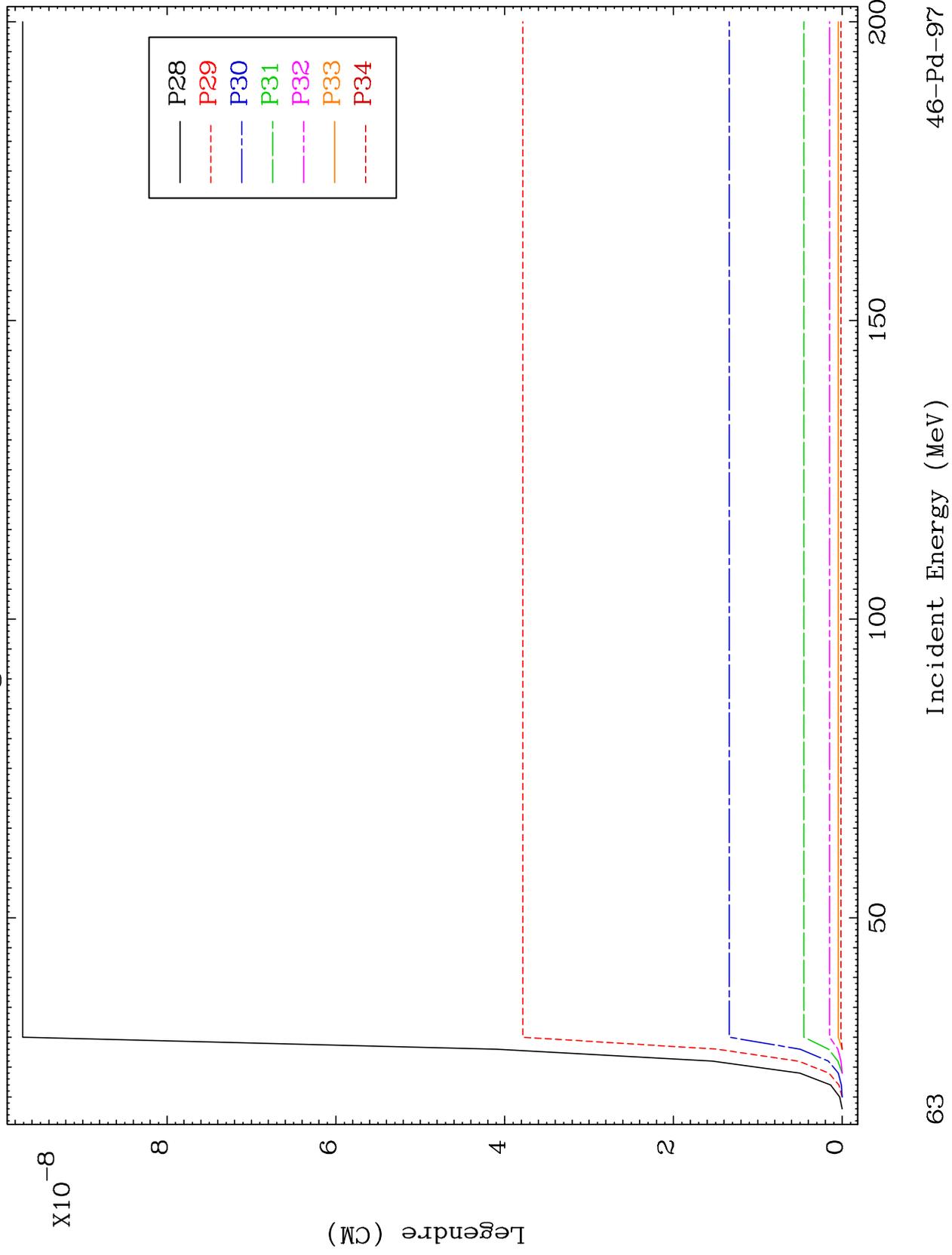
Incident Energy (MeV)

46-Pd-97

MAT 4610

MT= 64 (n,n') Level
Legendre Coefficients

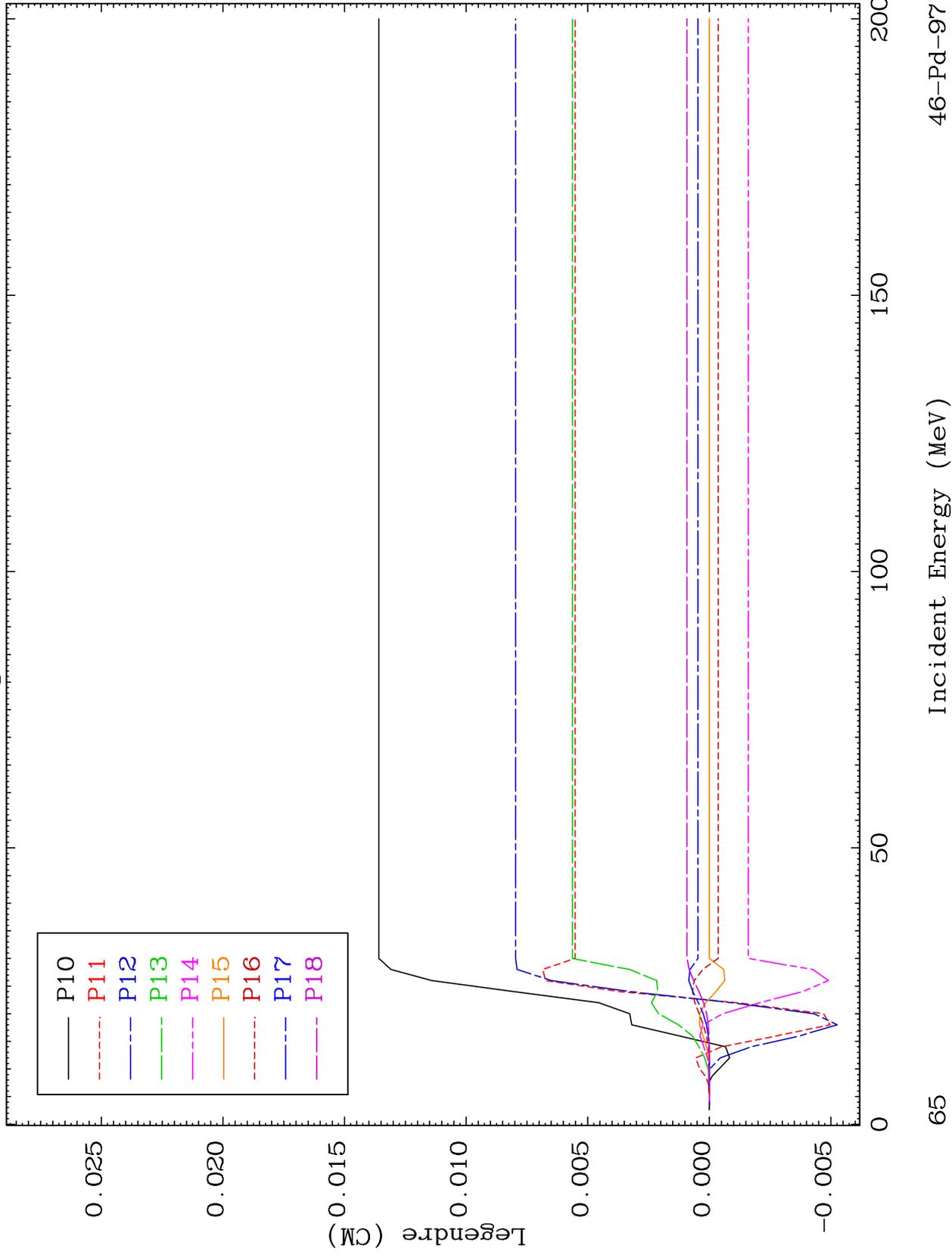
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MAT 4610

MT= 65 (n,n') Level
Legendre Coefficients

46-Pd-97



46-Pd-97

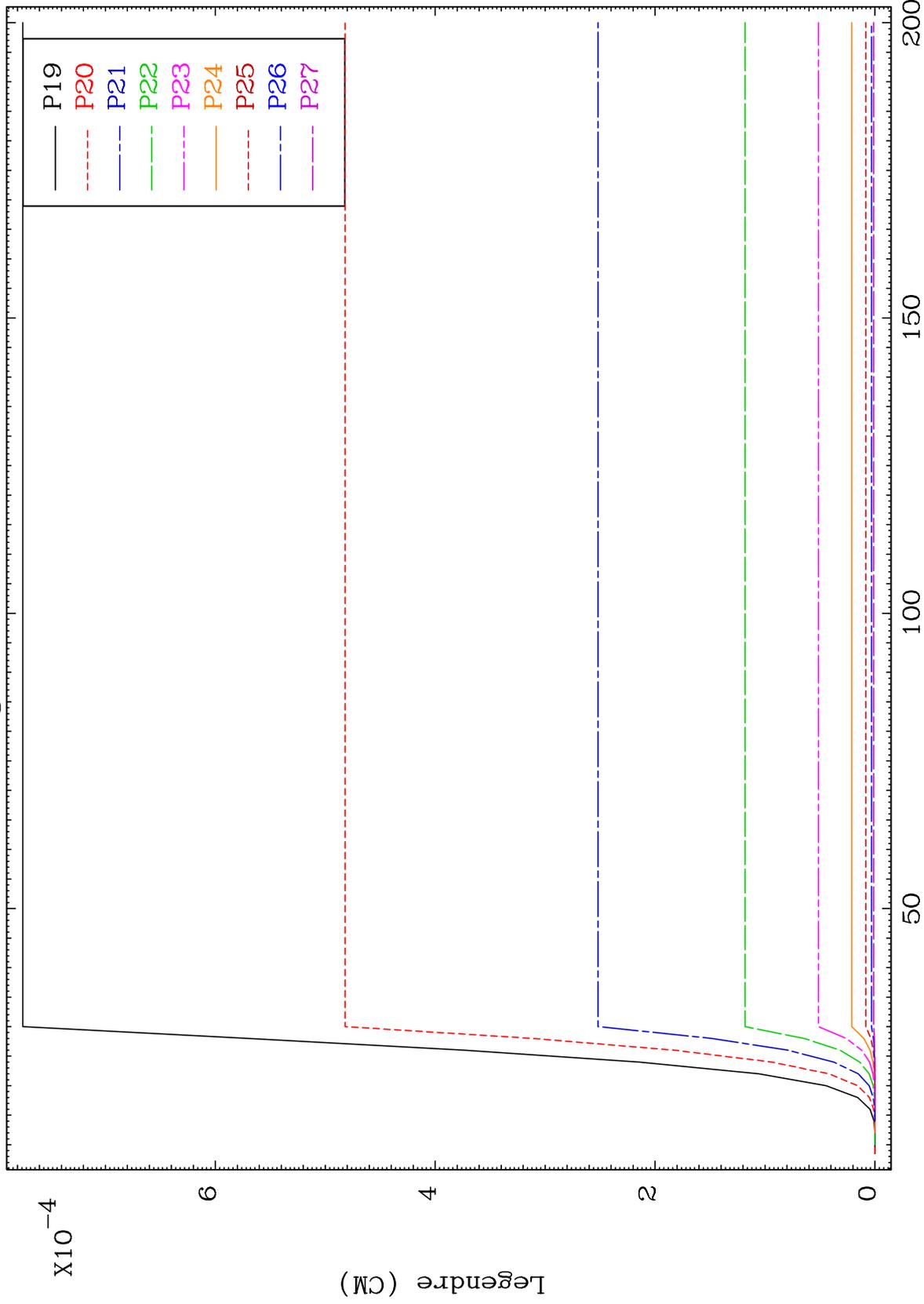
Incident Energy (MeV)

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MAT 4610

MT= 65 (n,n') Level
Legendre Coefficients

46-Pd-97



66

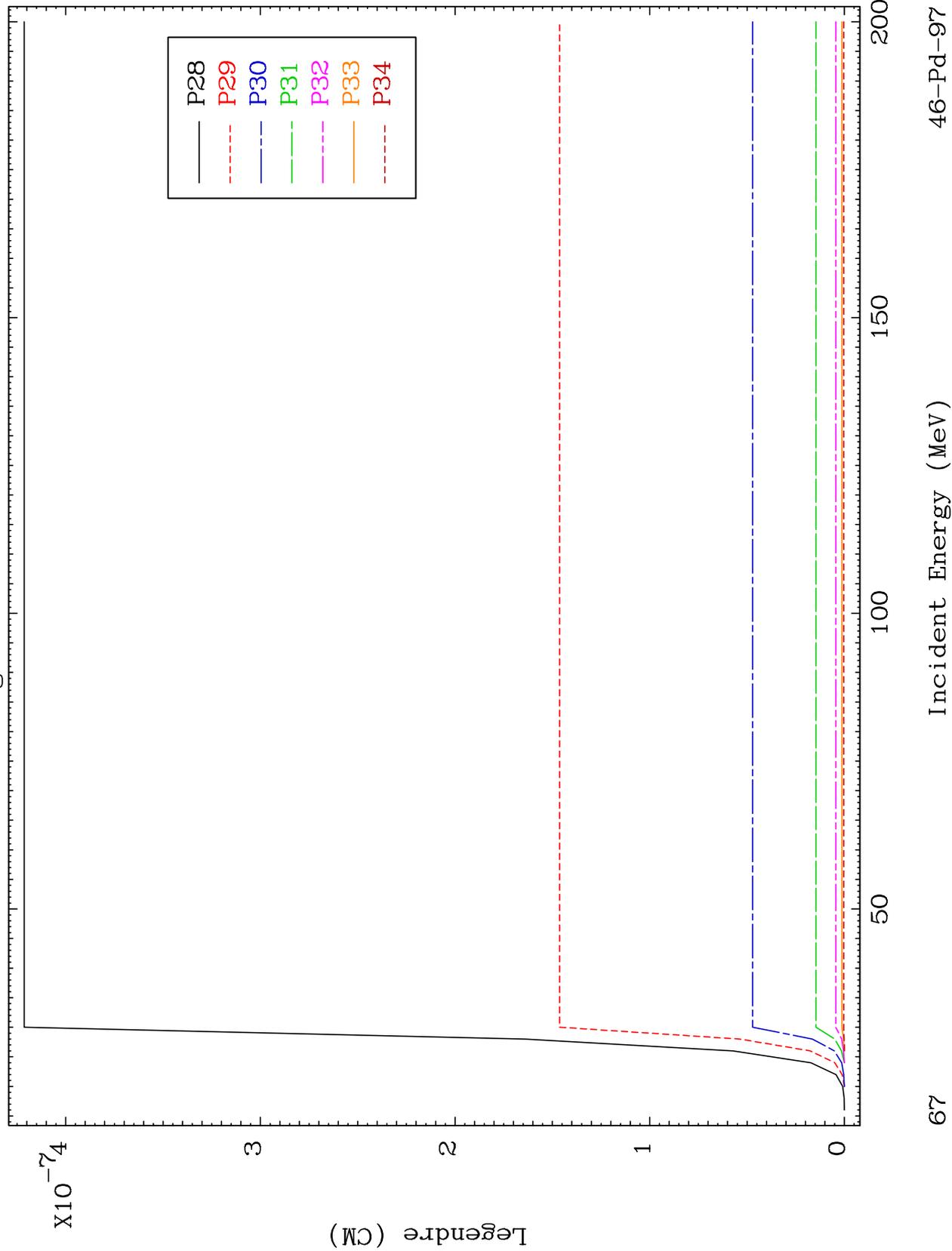
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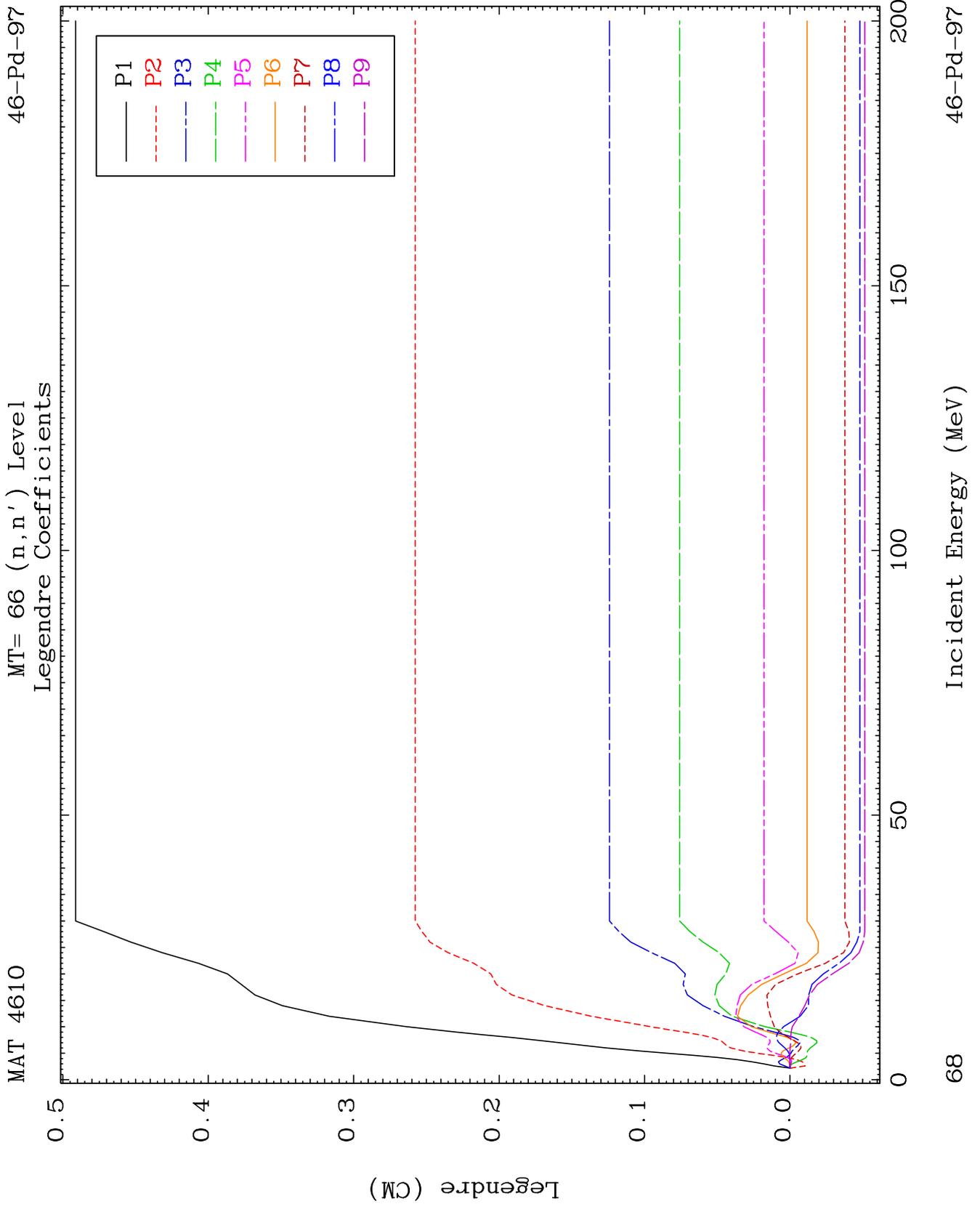
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MAT 4610

MT= 65 (n,n') Level
Legendre Coefficients

46-Pd-97

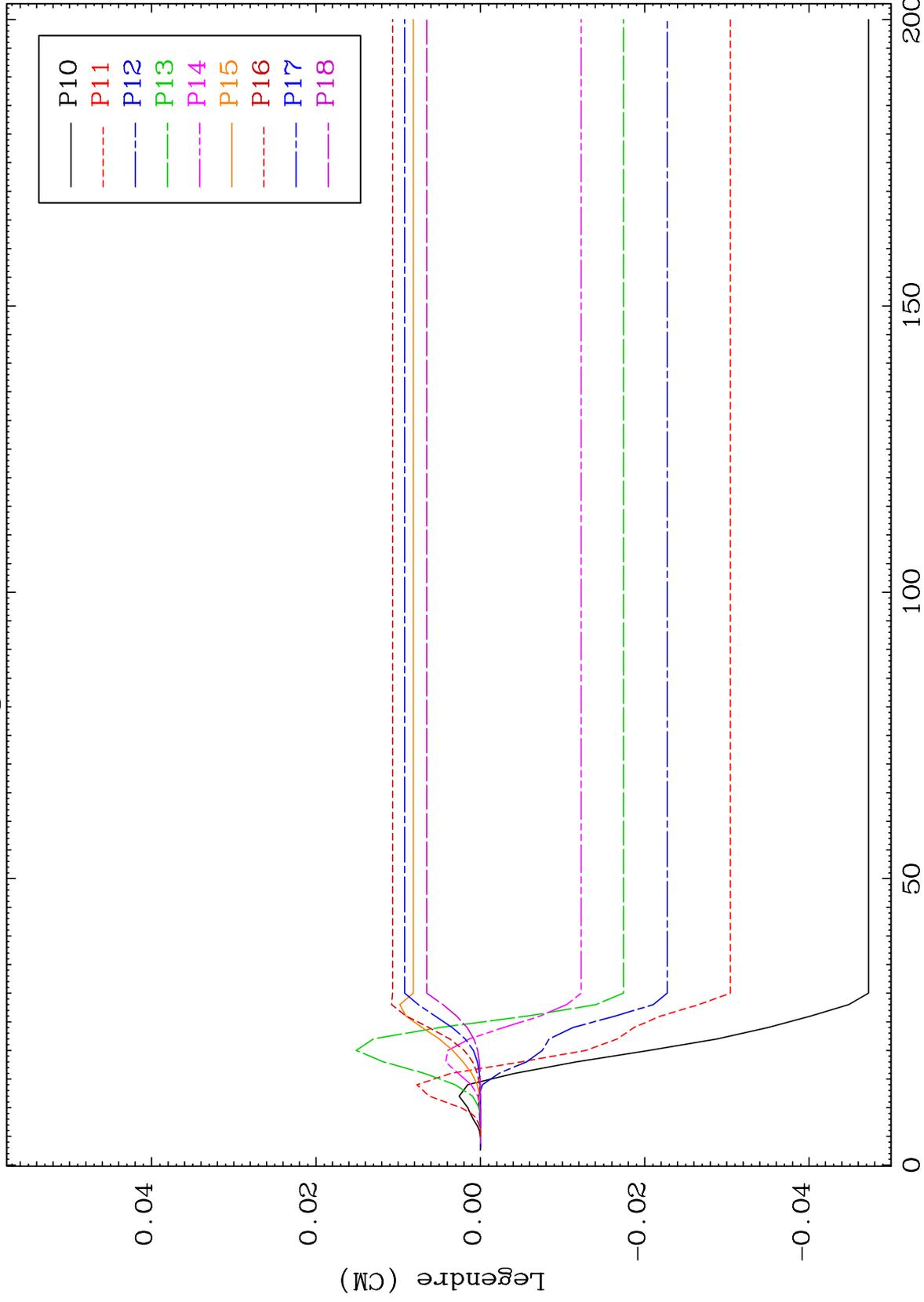




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MT= 66 (n,n') Level
Legendre Coefficients

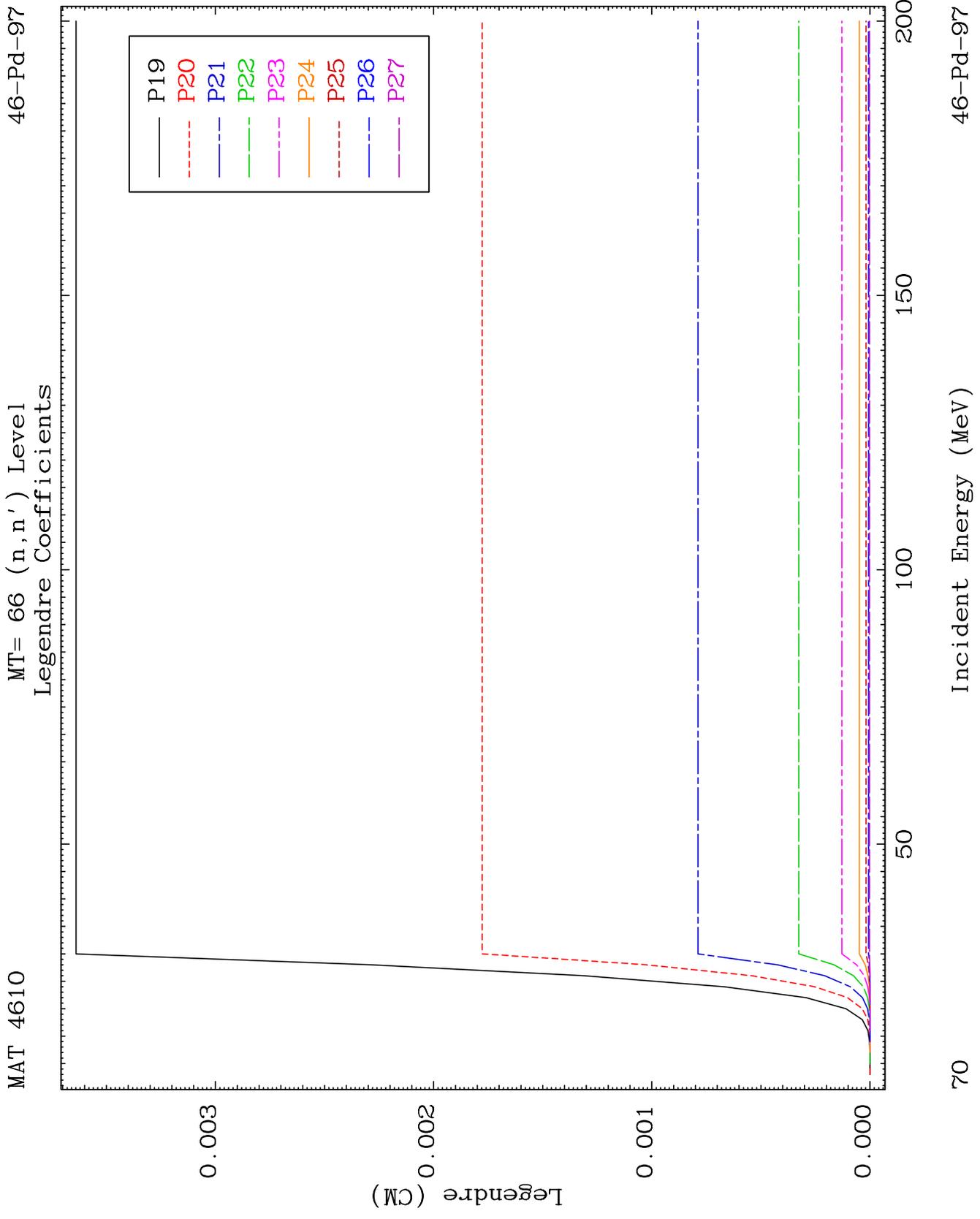
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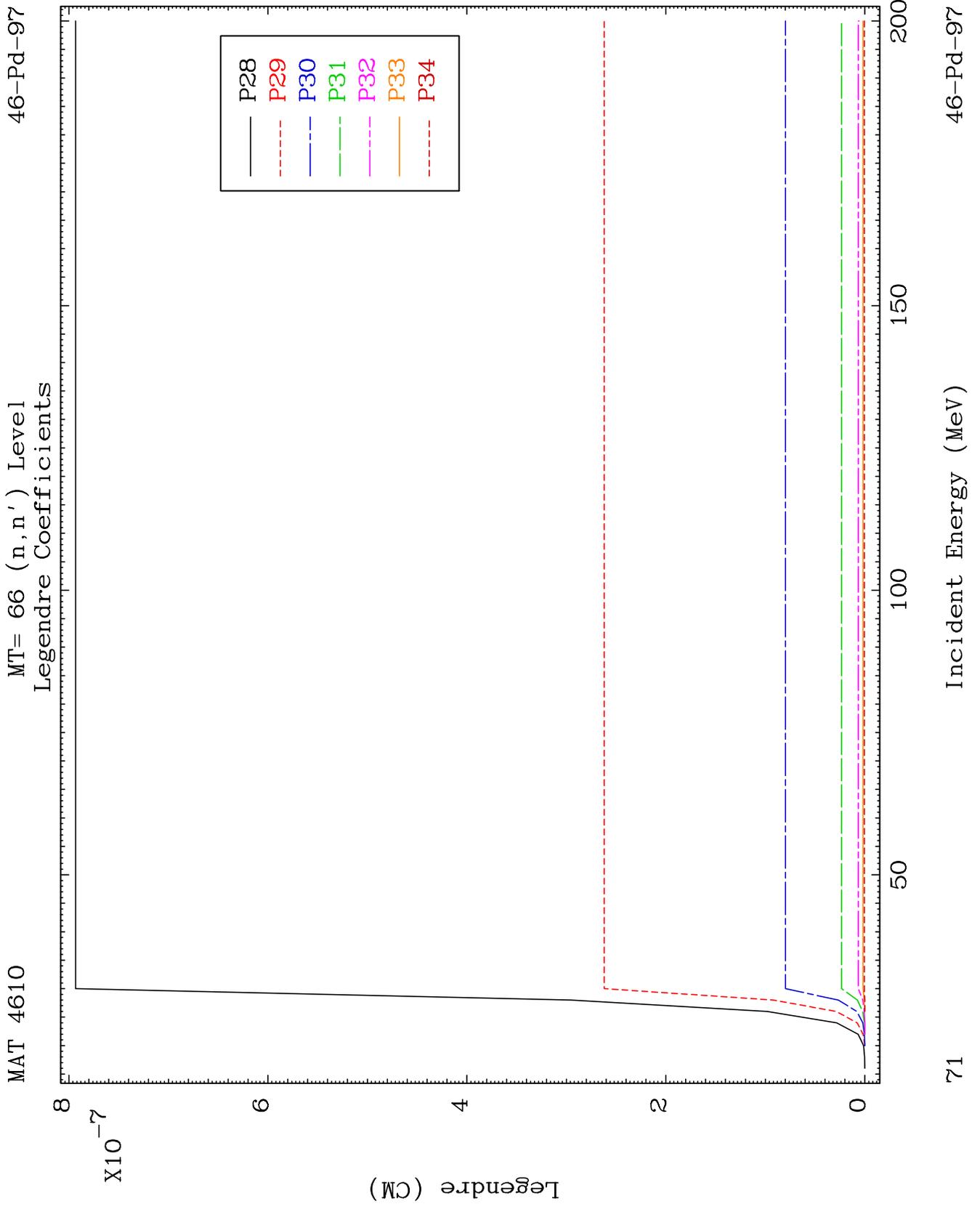


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Incident Energy (MeV)

46-Pd-97



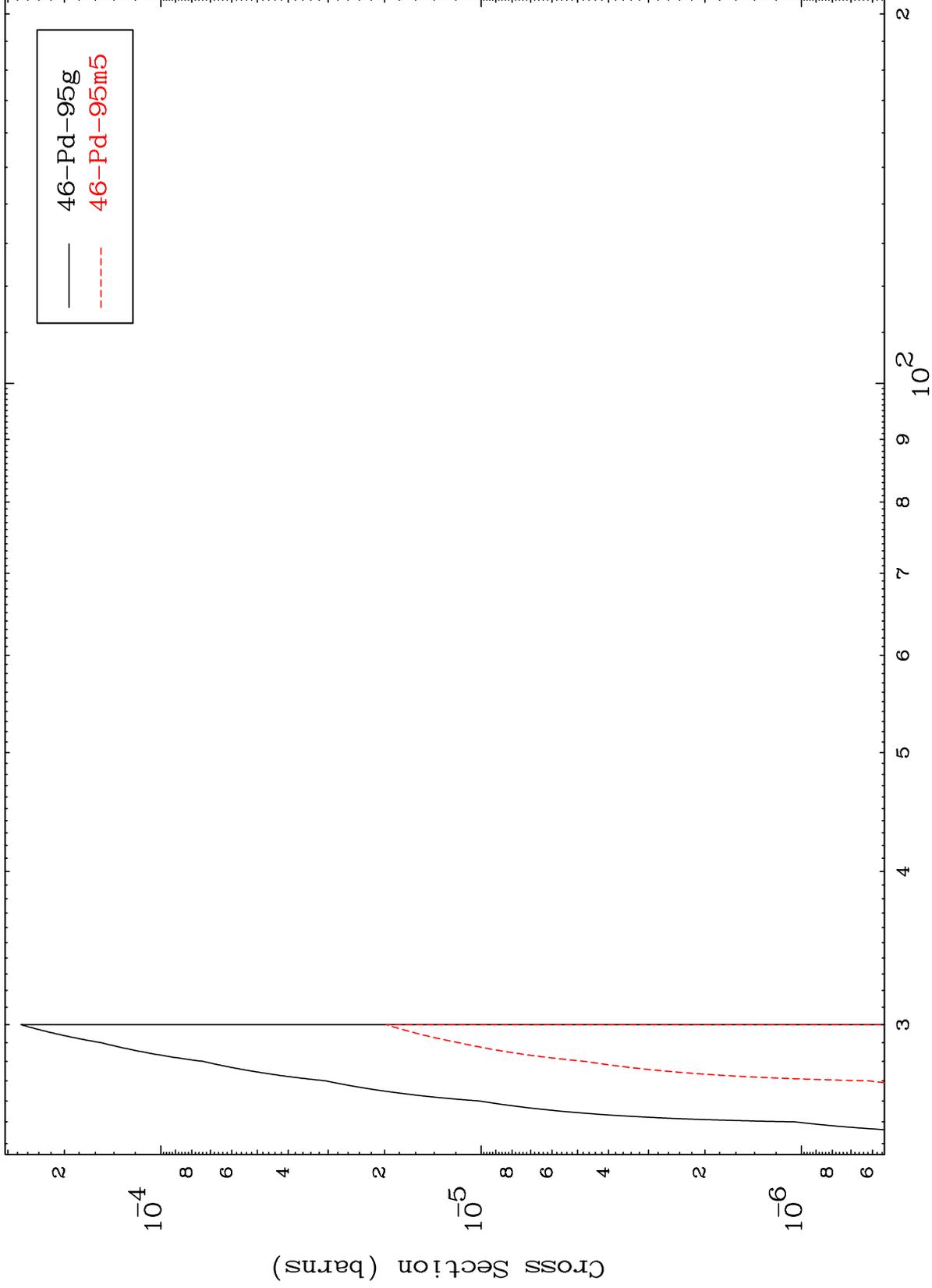


MAT 4610

(n,3n)

46-Pd-97

Radionuclide Production Cross Section



72

Incident Energy (MeV)

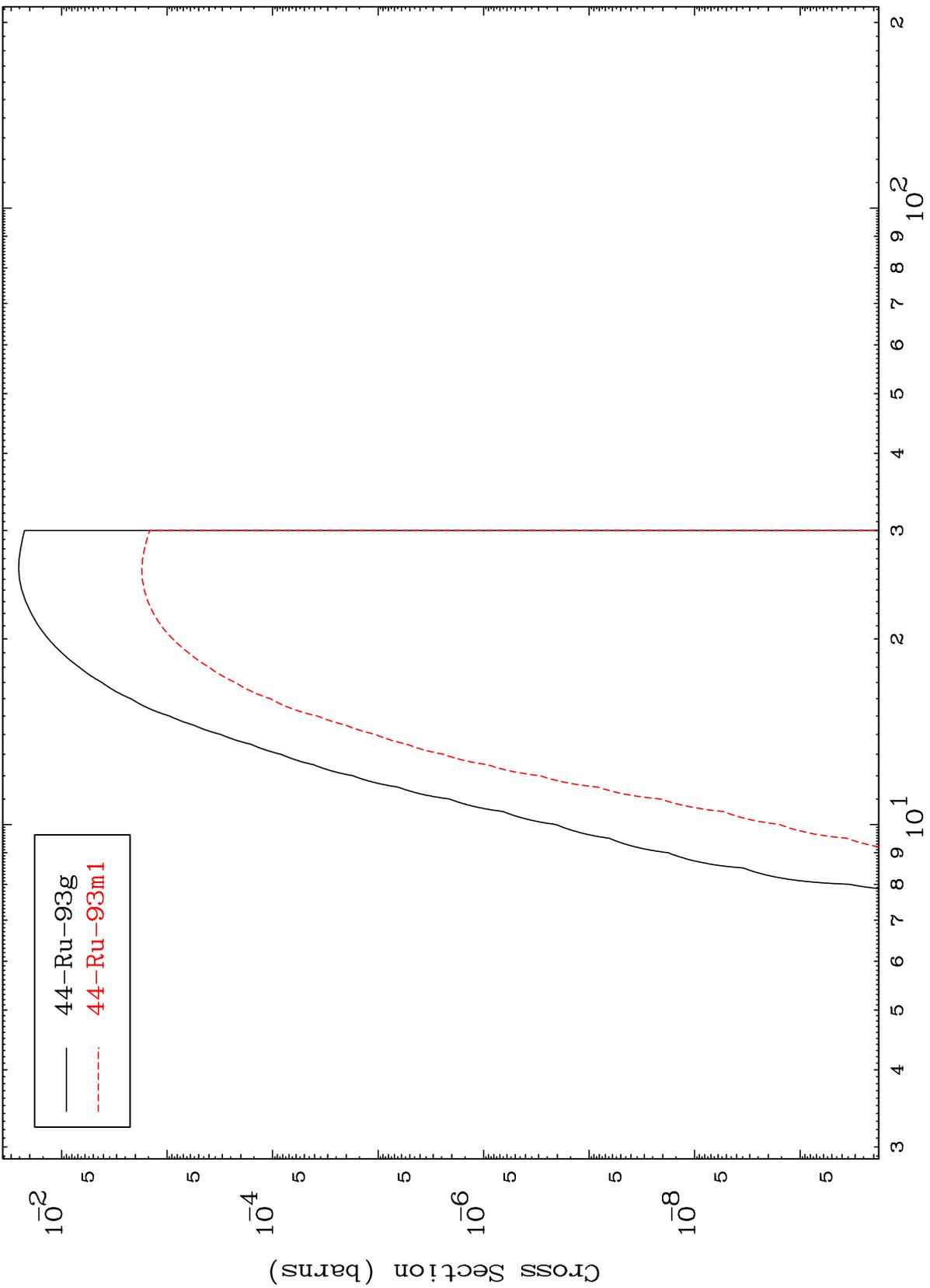
46-Pd-97

MAT 4610

(n,n') α

46-Pd-97

Radionuclide Production Cross Section



— 44-Ru-93g
- - - 44-Ru-93m1

73

Incident Energy (MeV)

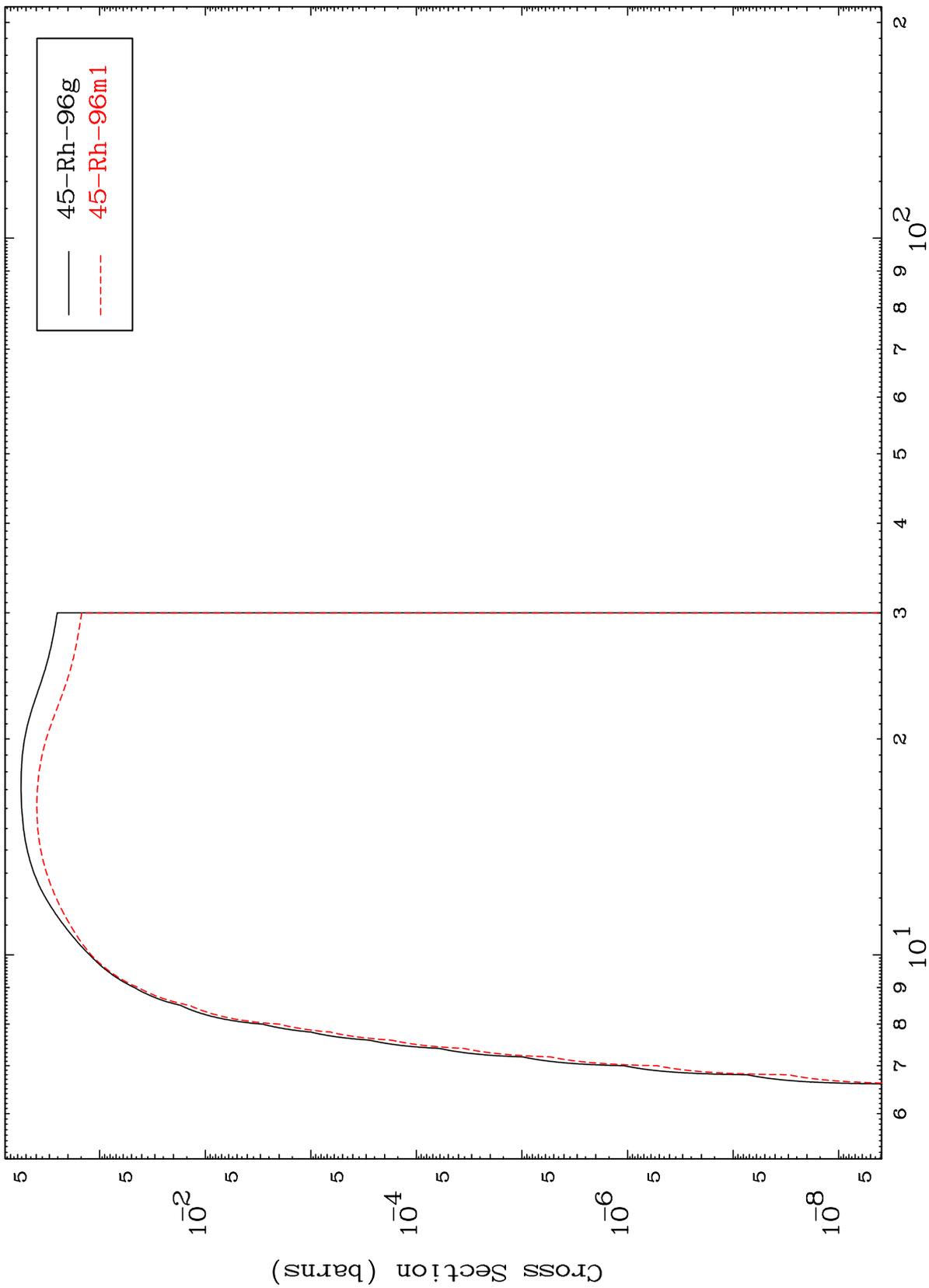
46-Pd-97

MAT 4610

(n,n') p

46-Pd-97

Radionuclide Production Cross Section



74

Incident Energy (MeV)

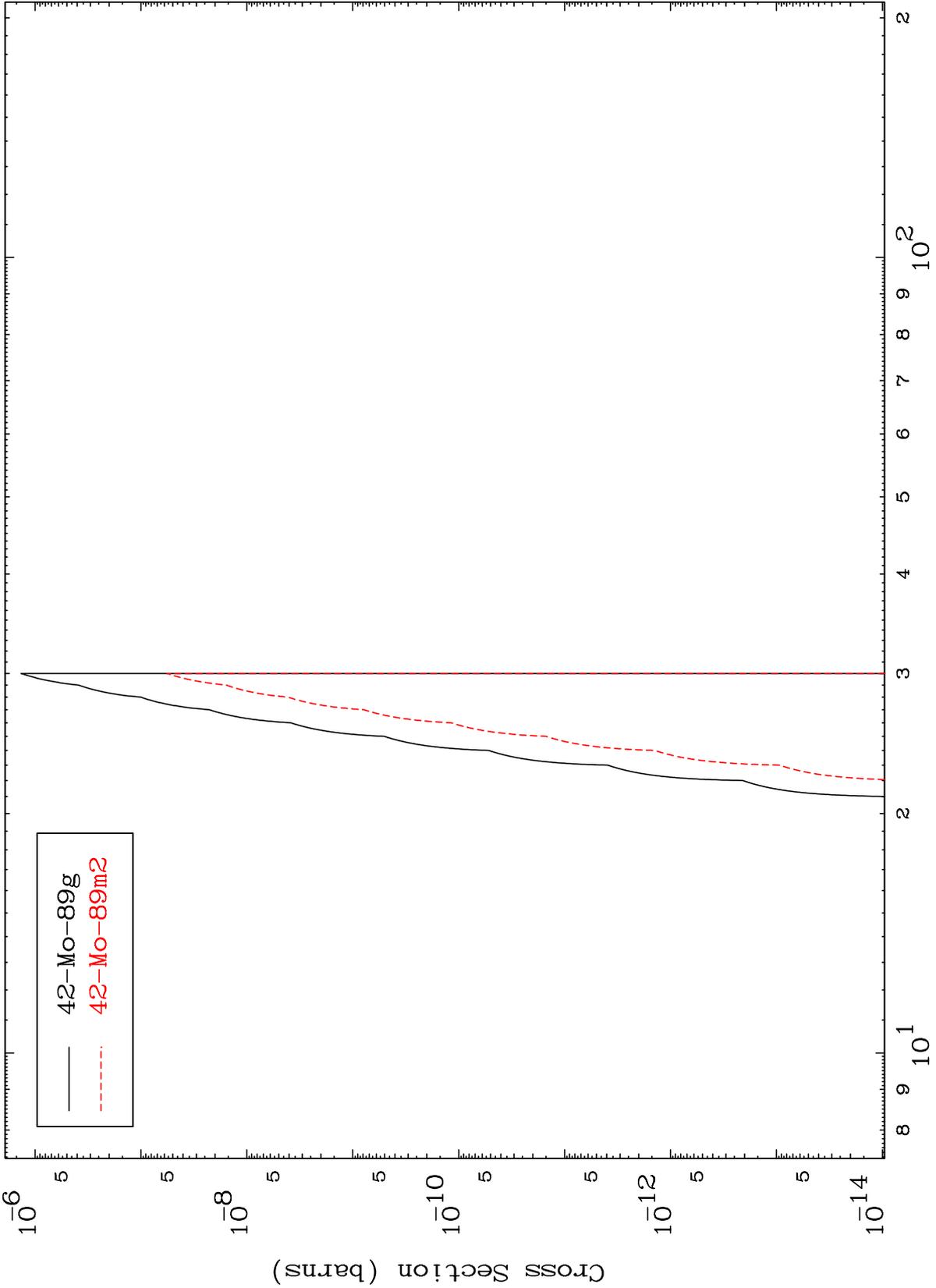
46-Pd-97

MAT 4610

(n,n') 2α

46-Pd-97

Radionuclide Production Cross Section



75

Incident Energy (MeV)

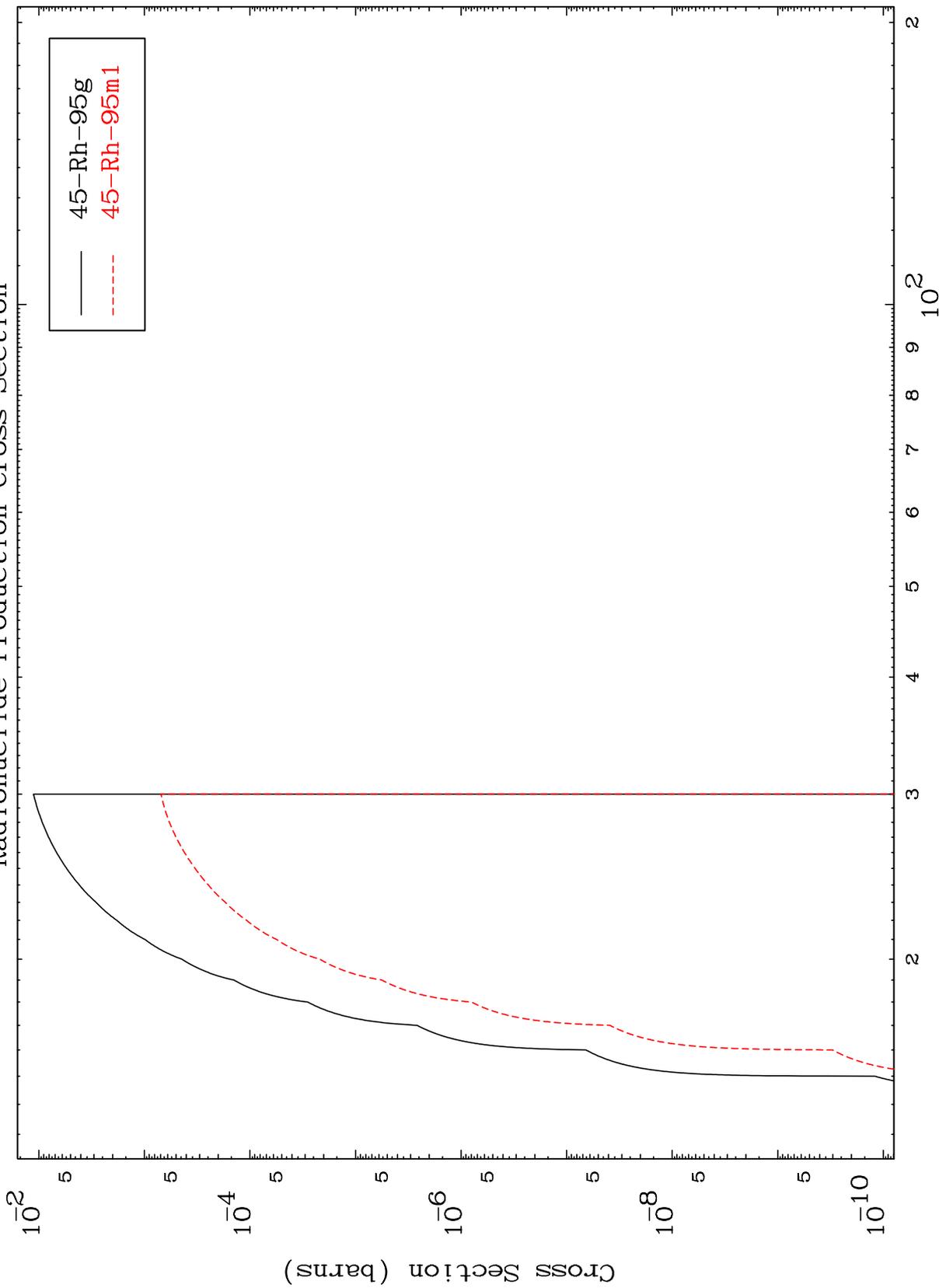
46-Pd-97

MAT 4610

(n,n') d

46-Pd-97

Radionuclide Production Cross Section



76

Incident Energy (MeV)

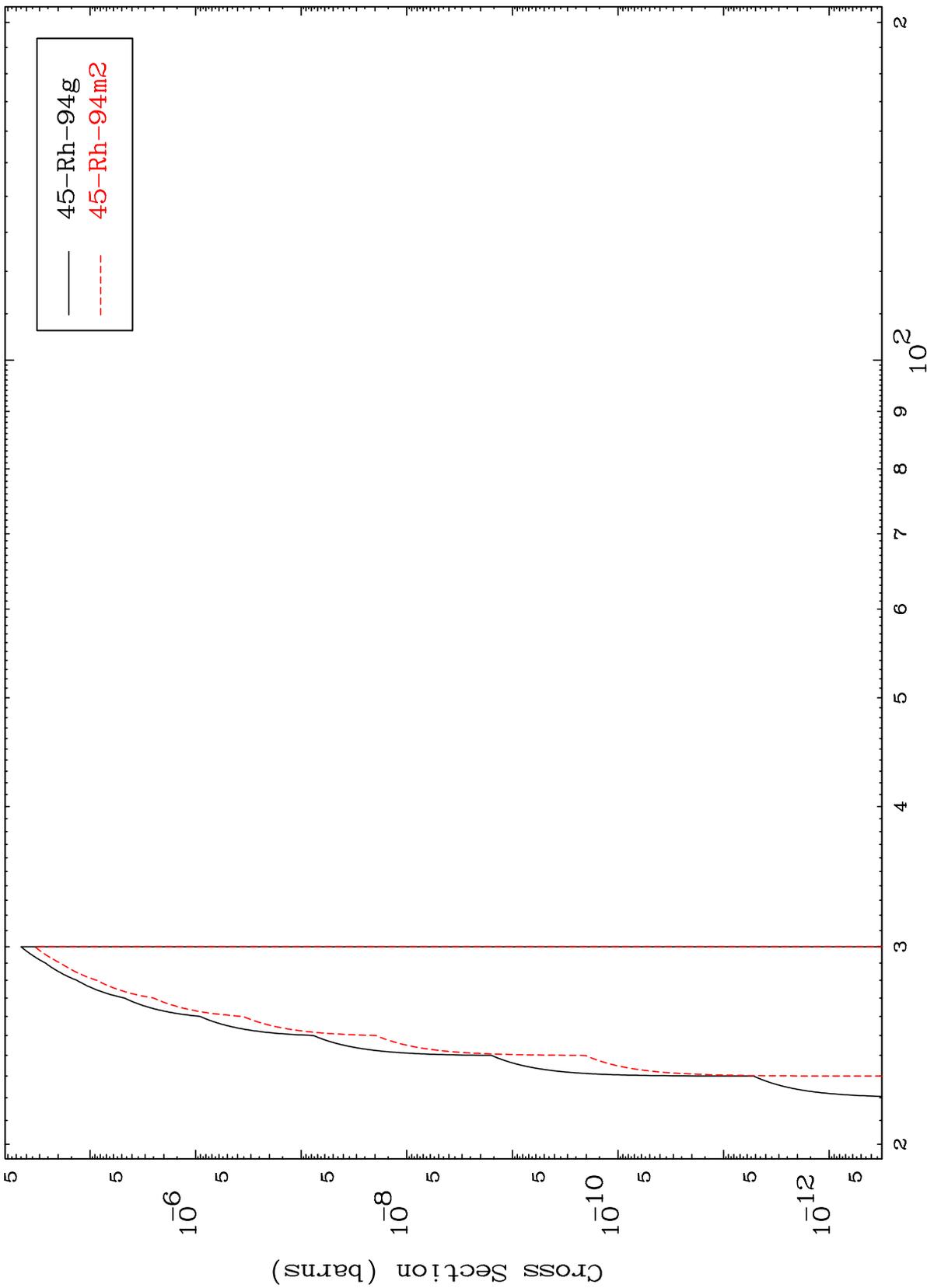
46-Pd-97

MAT 4610

(n,n') t

46-Pd-97

Radionuclide Production Cross Section



77

Incident Energy (MeV)

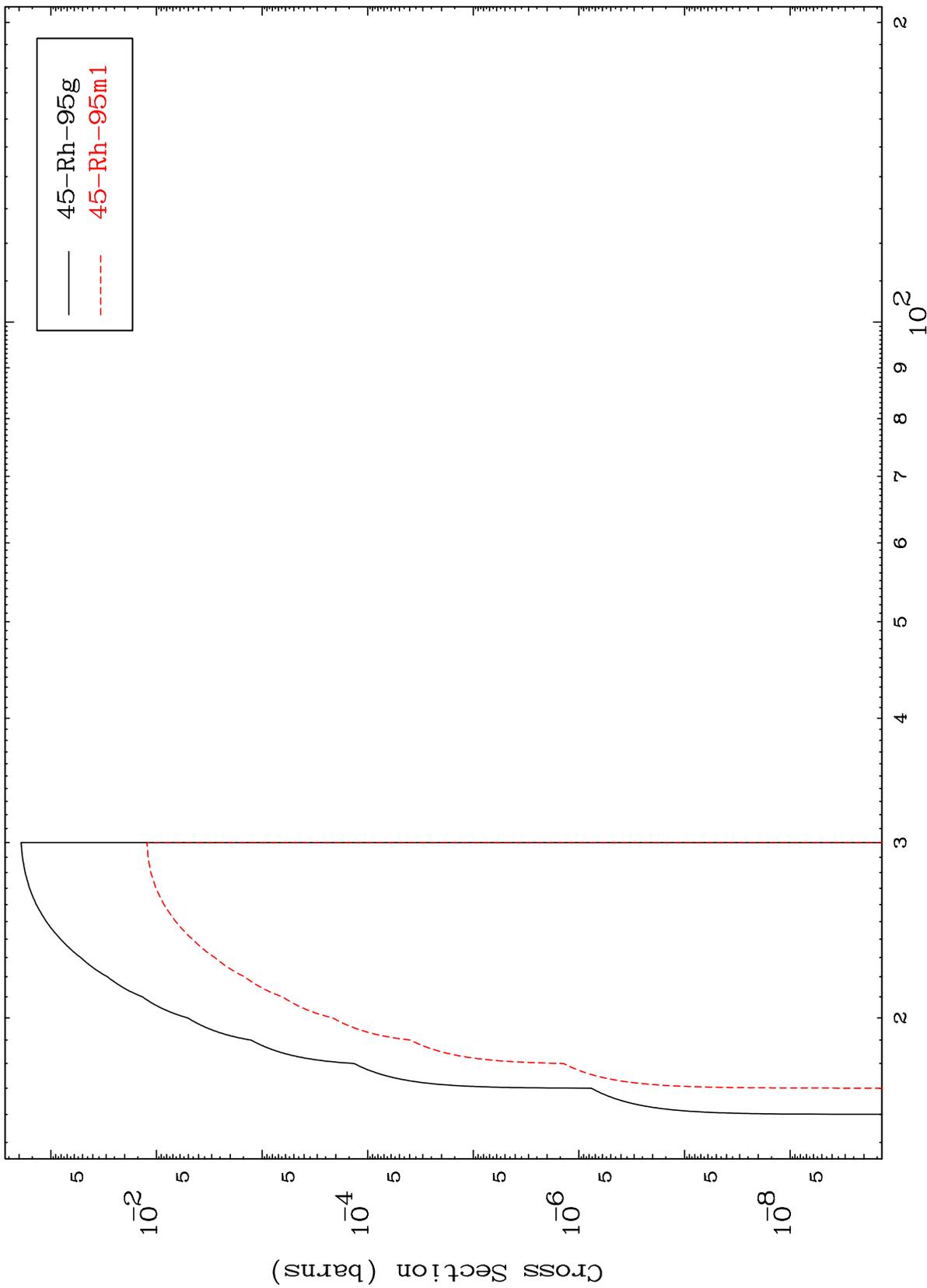
46-Pd-97

MAT 4610

(n,2n) p

46-Pd-97

Radionuclide Production Cross Section



78

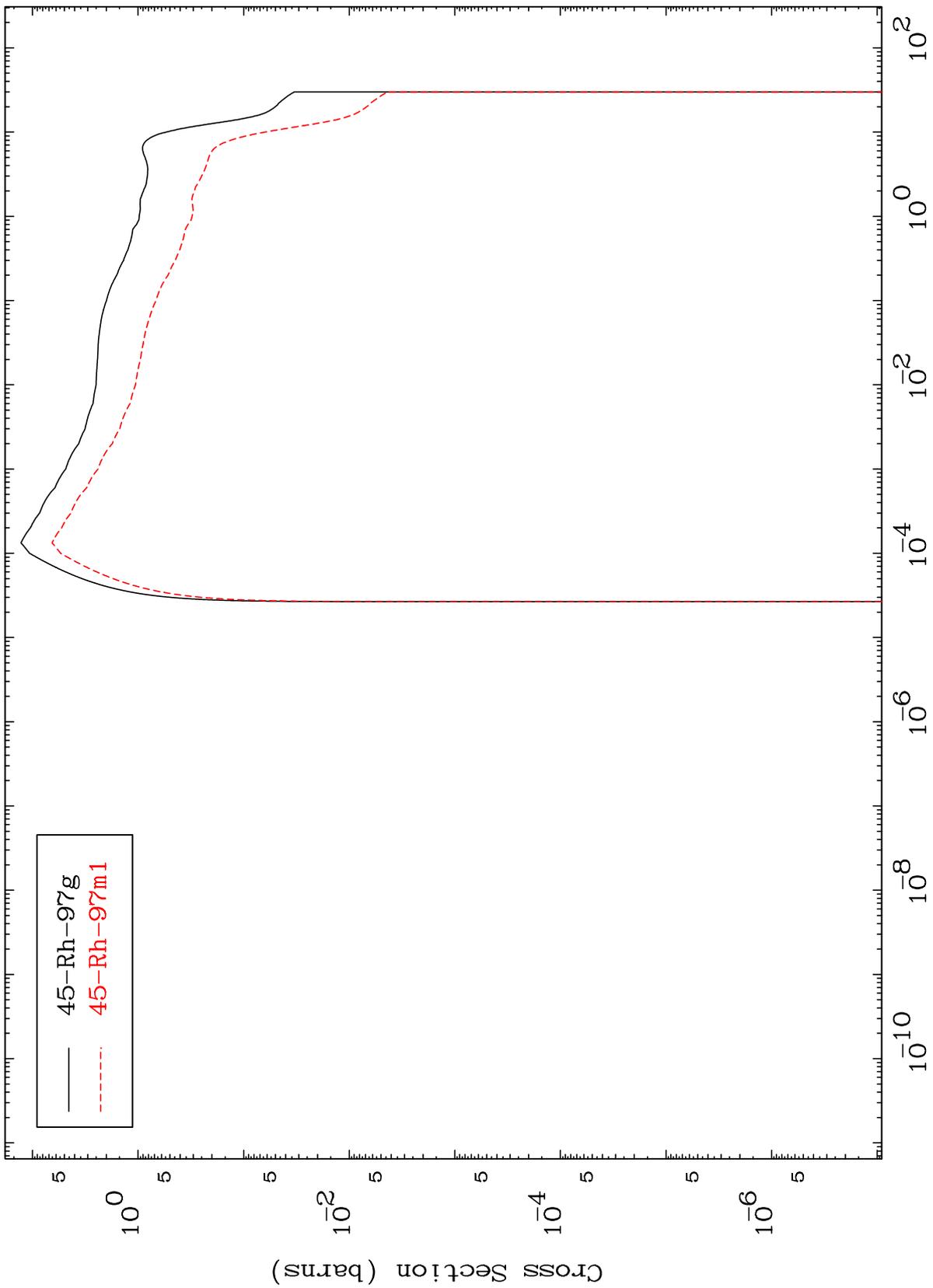
Incident Energy (MeV)

46-Pd-97

MAT 4610

46-Pd-97

(n,p)
Radionuclide Production Cross Section



— 45-Rh-97g
- - - 45-Rh-97m1

79

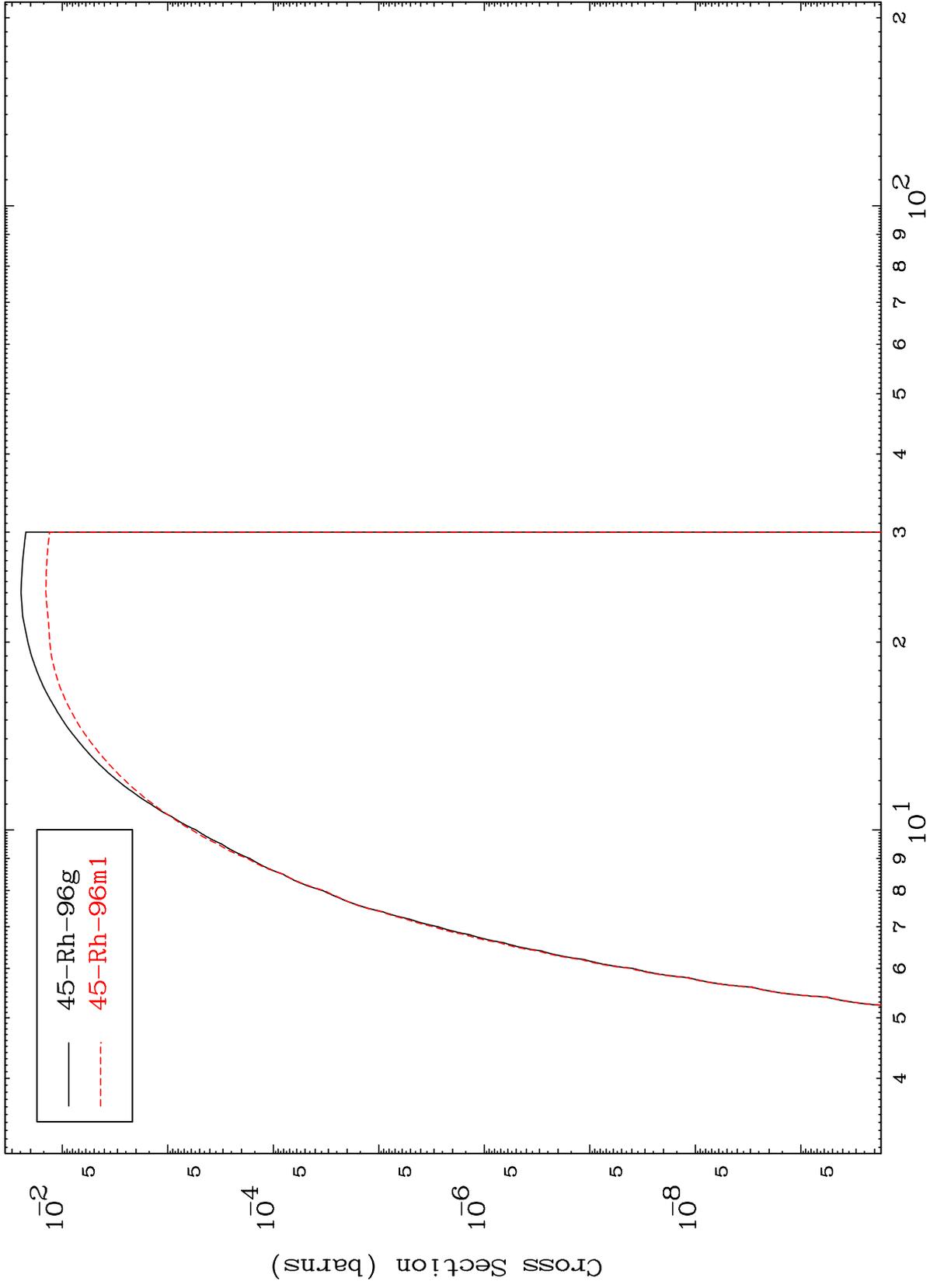
46-Pd-97

Incident Energy (MeV)

MAT 4610

46-Pd-97

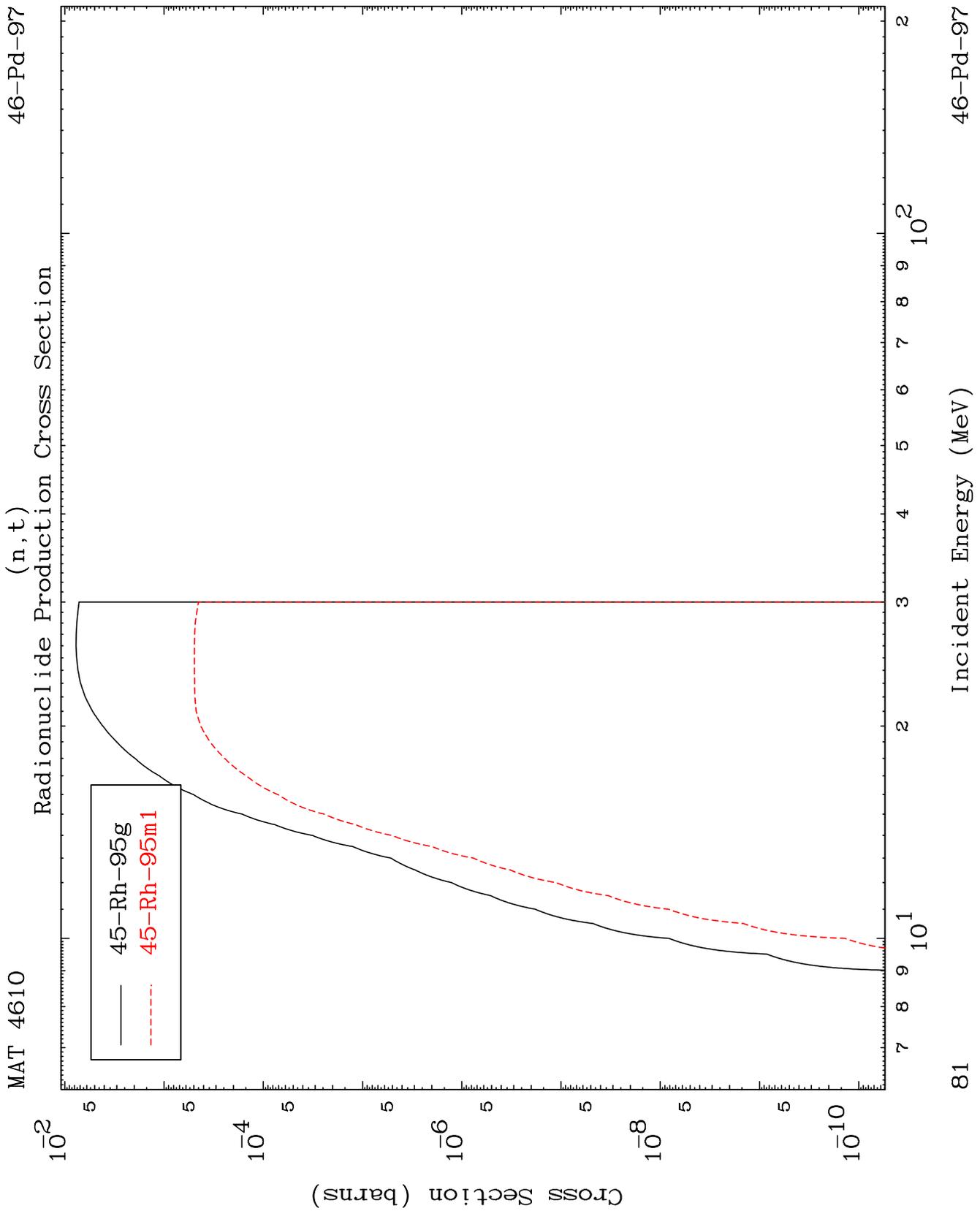
(n,d)
Radionuclide Production Cross Section



80

Incident Energy (MeV)

46-Pd-97



MAT 4610

(n,p) α

46-Pd-97

Radionuclide Production Cross Section

