

Program EVALPLOT
(Version 2021-1)

by

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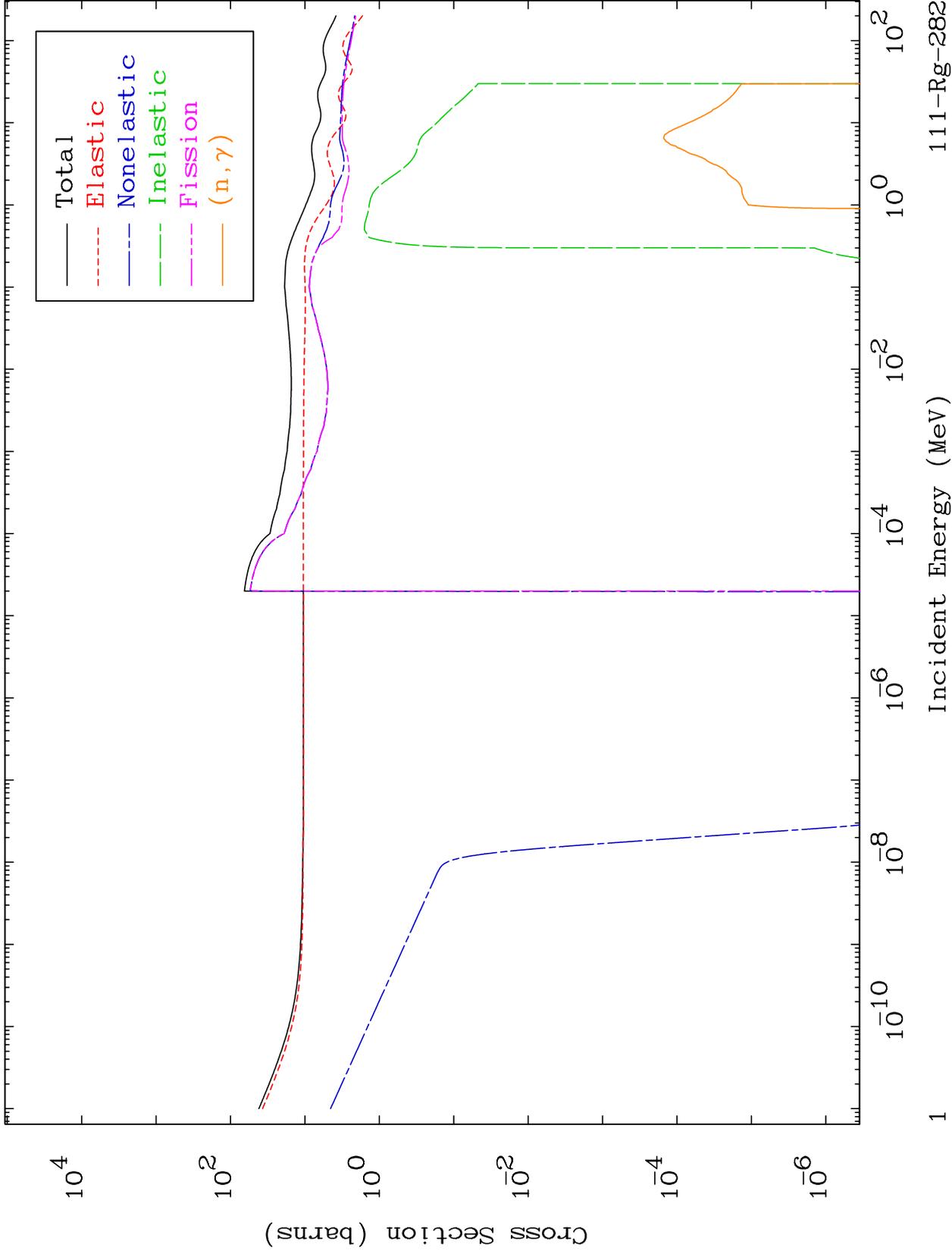
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Press Mouse Button to Start

MAT 1182

Neutron Major
293 Kelvin Cross Sections

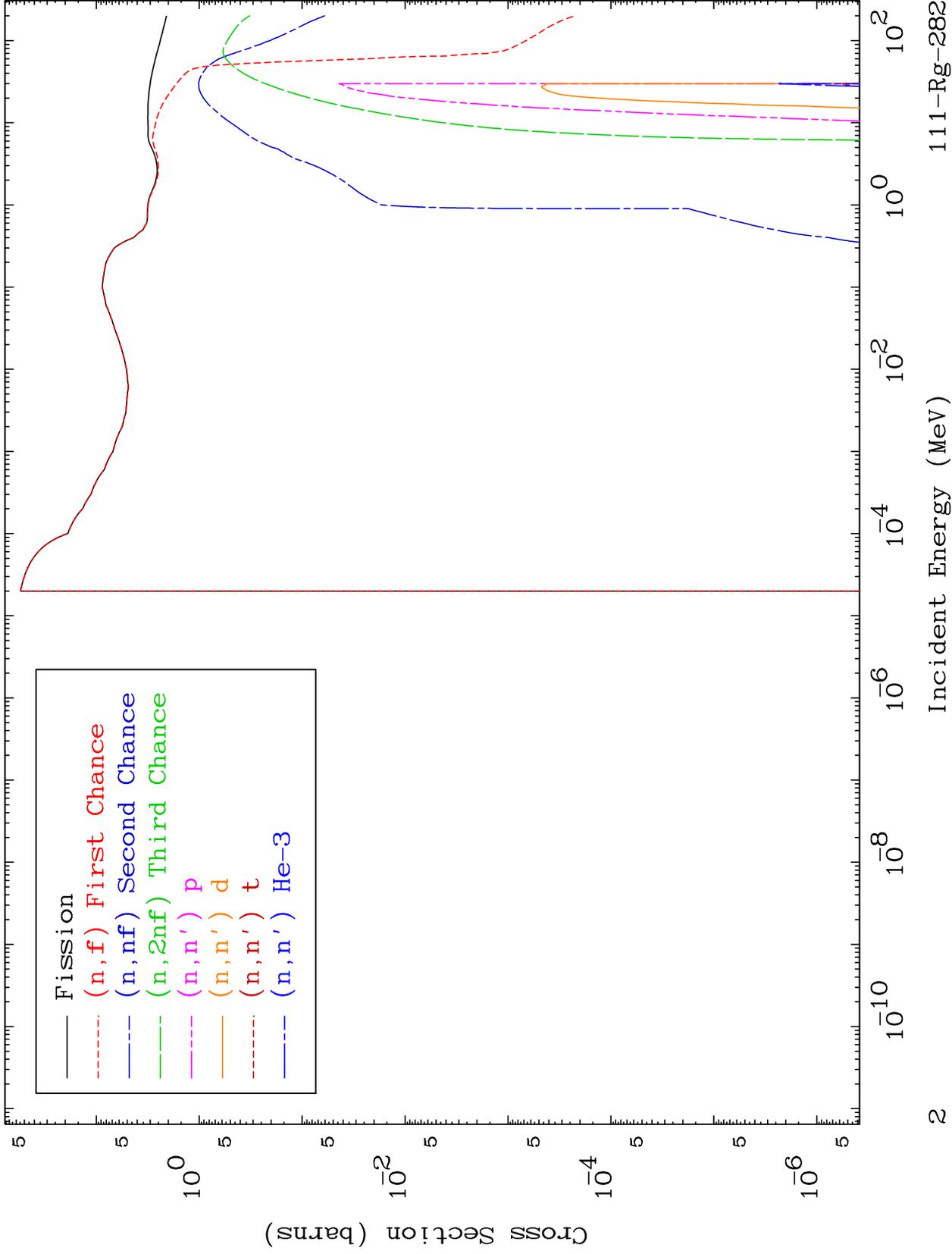
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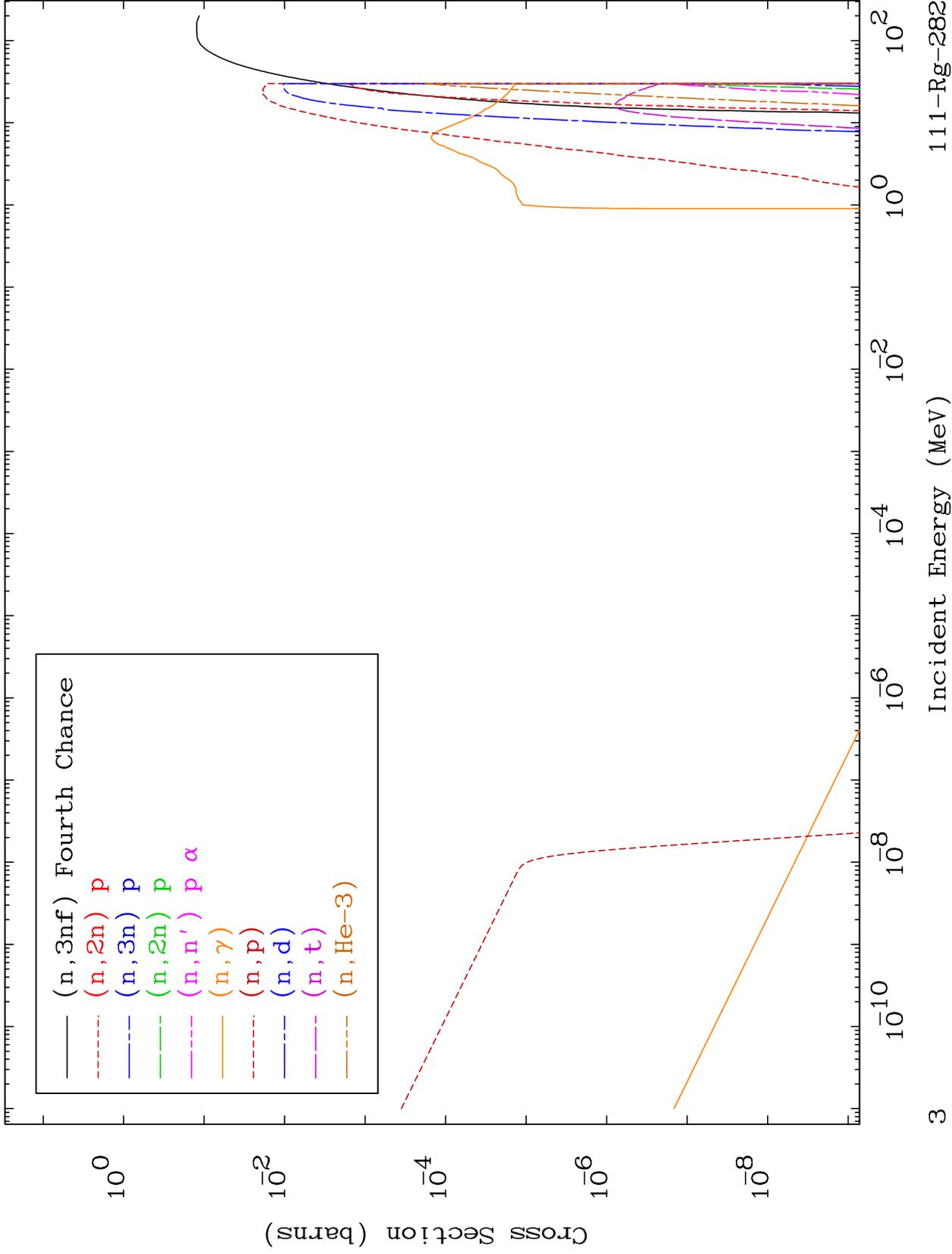


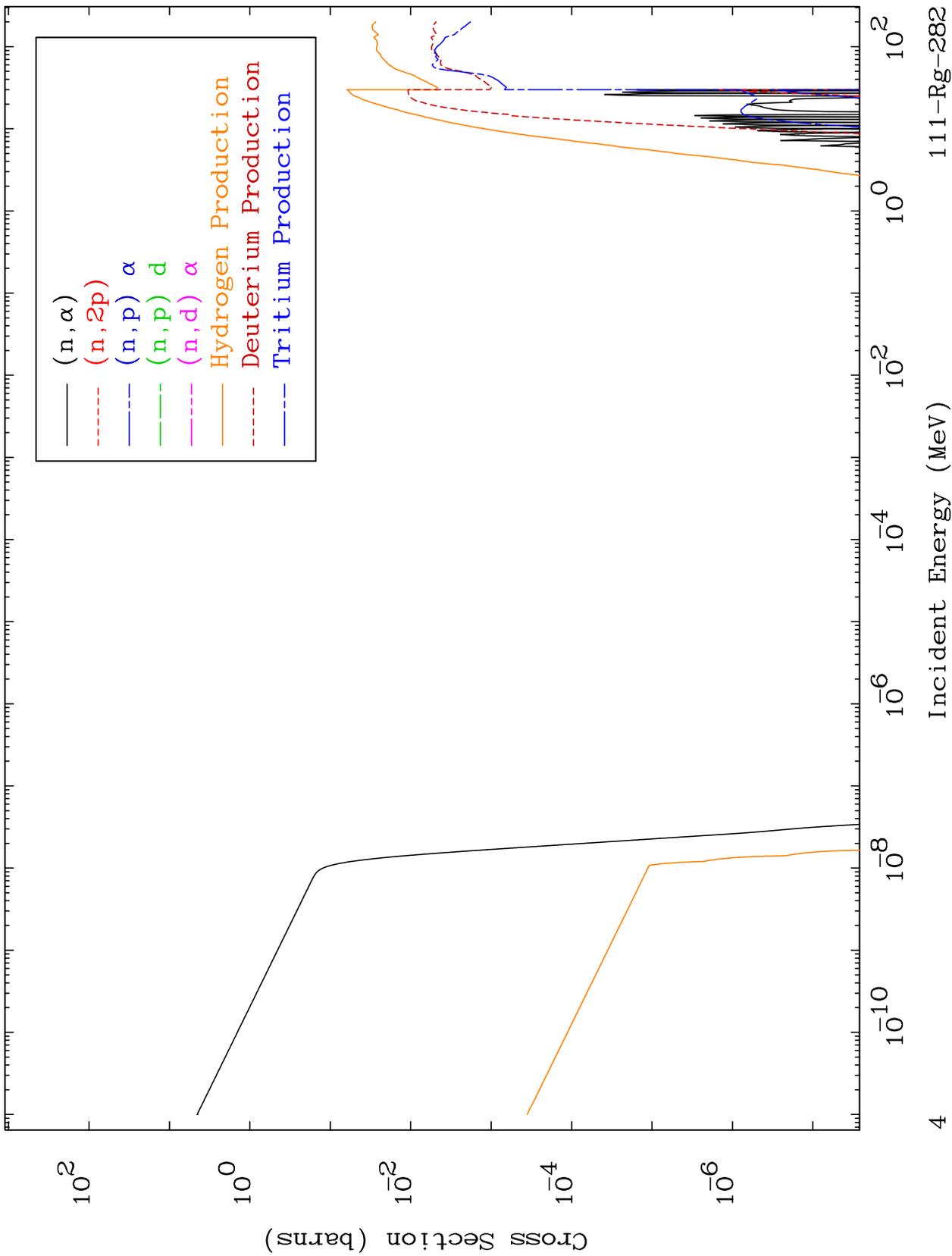
MAT 1182

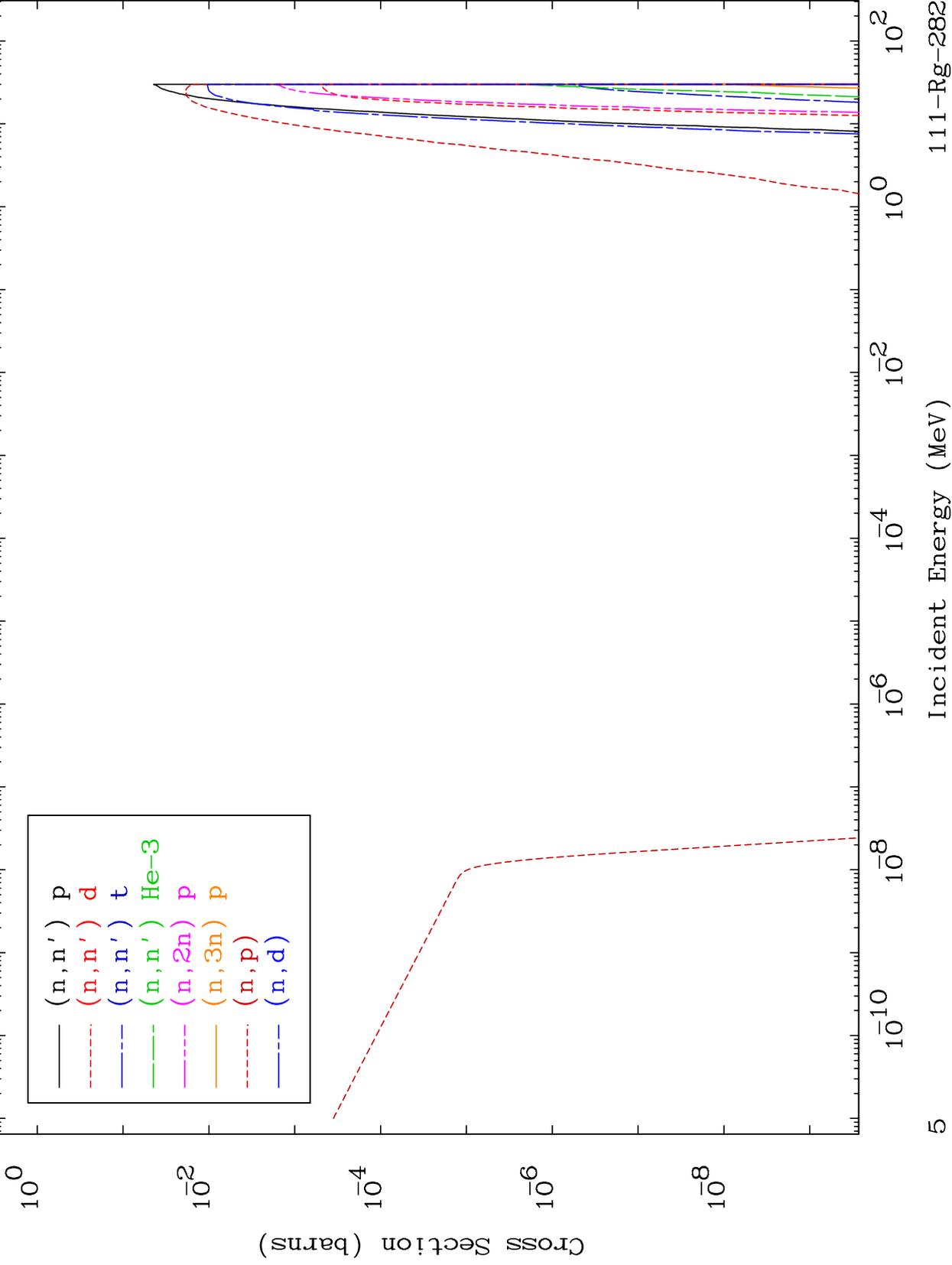
Neutron Absorption
293 Kelvin Cross Sections

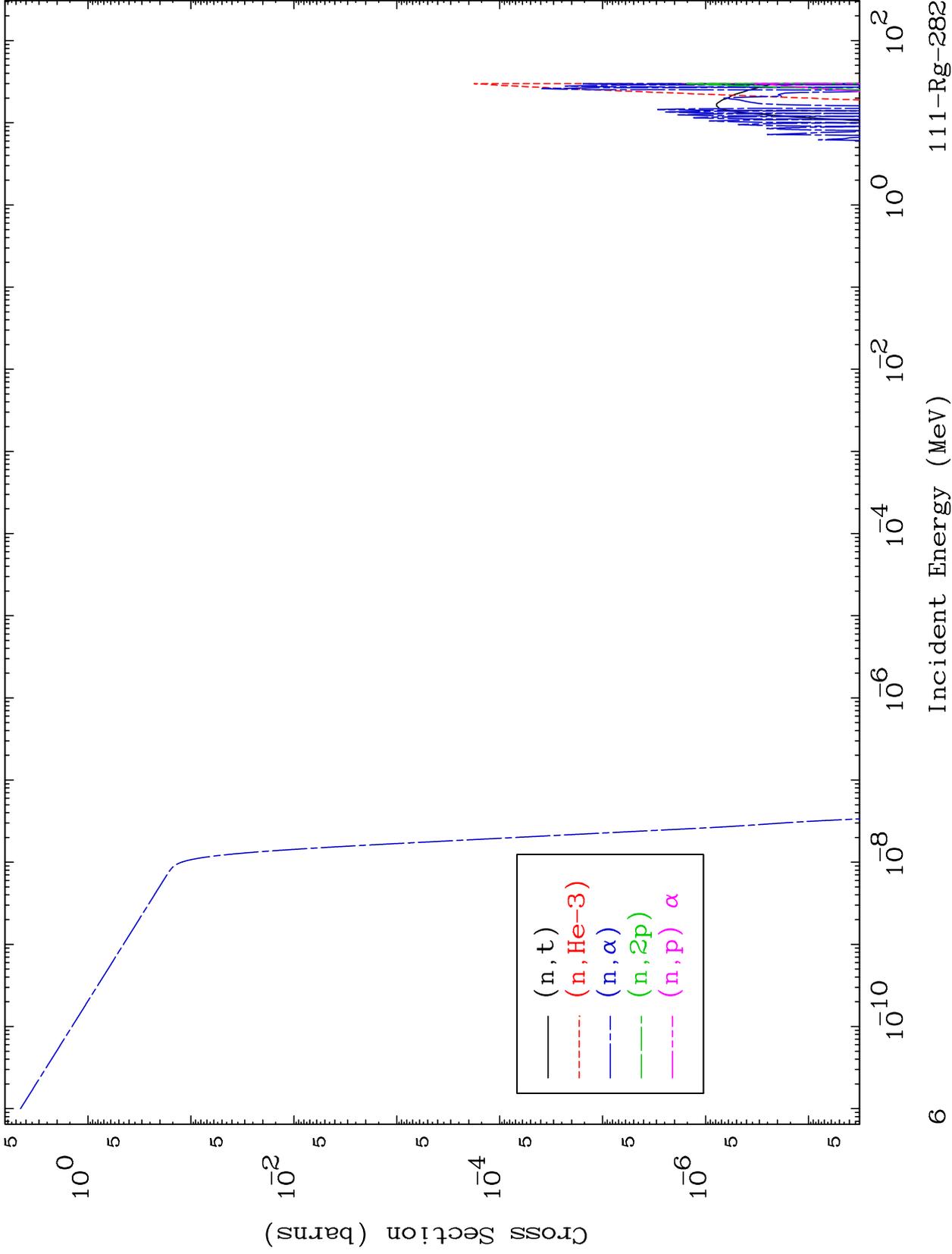
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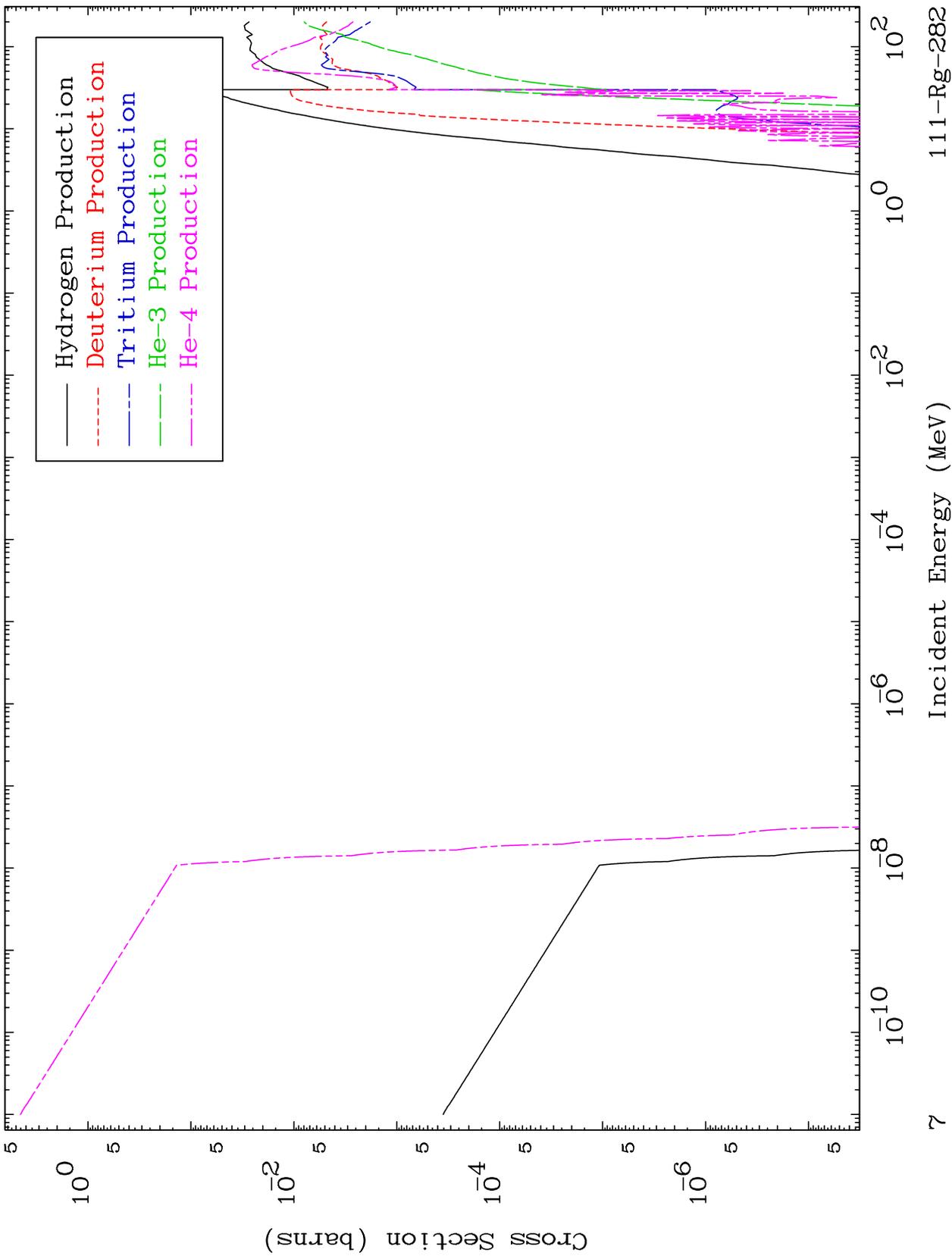








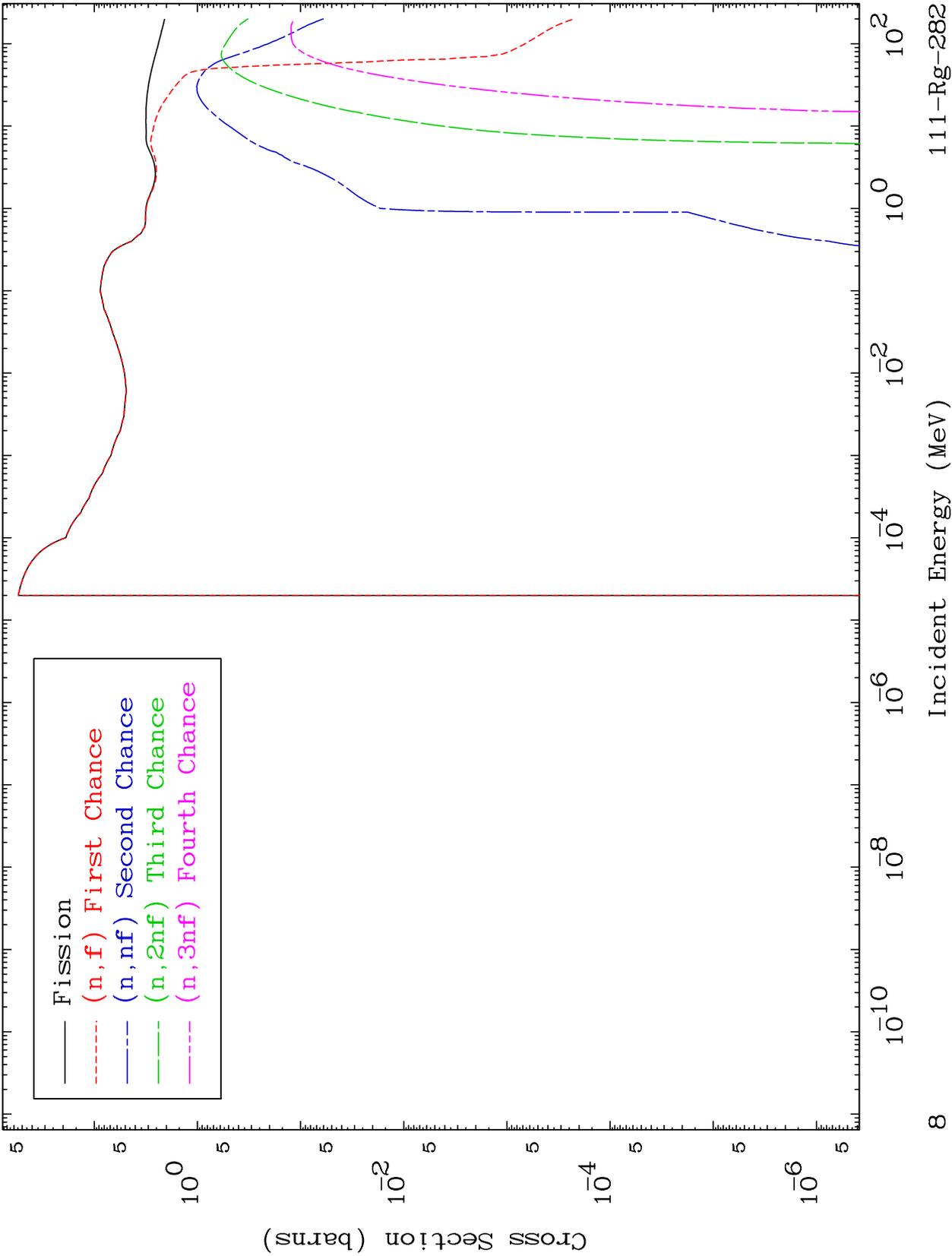


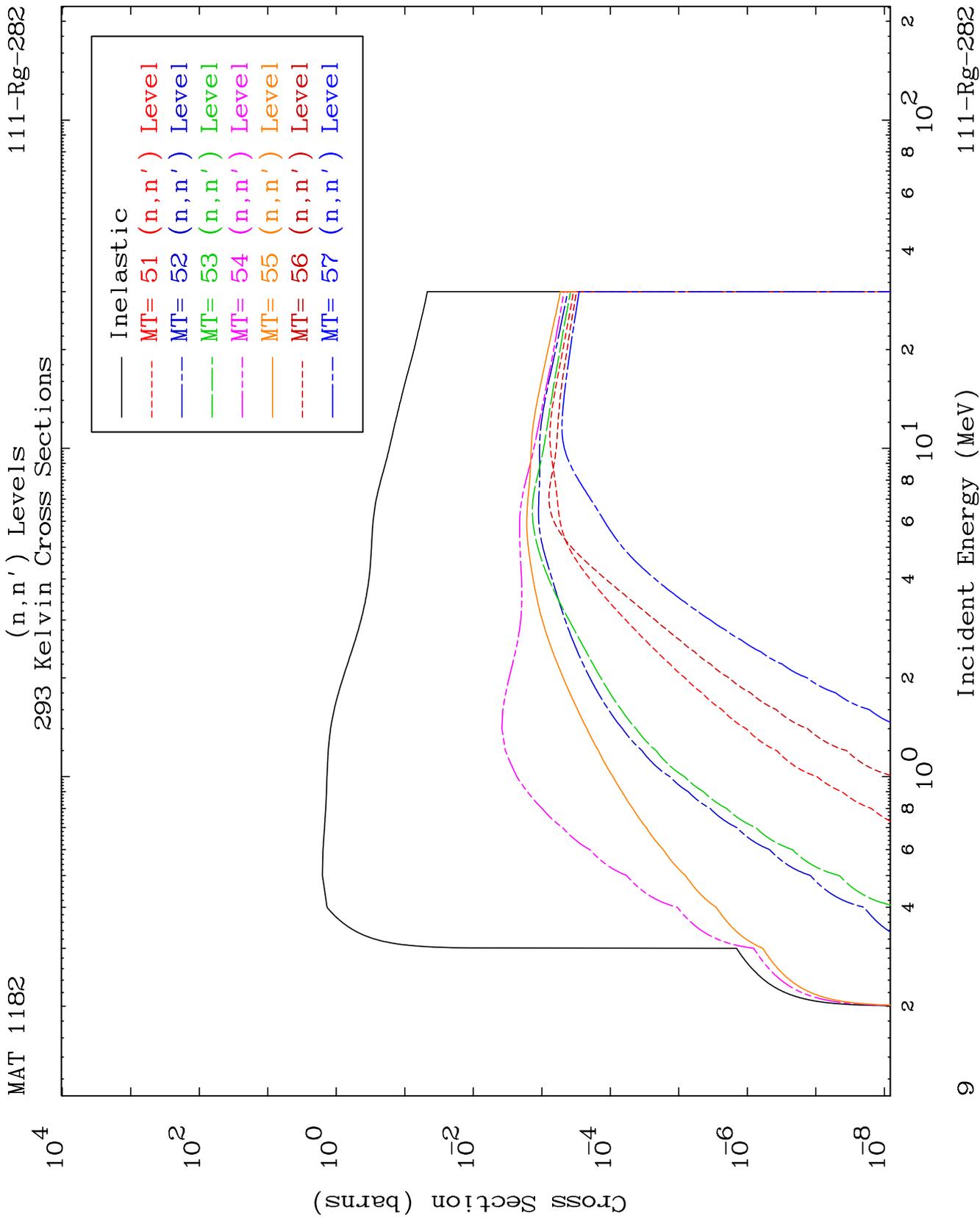


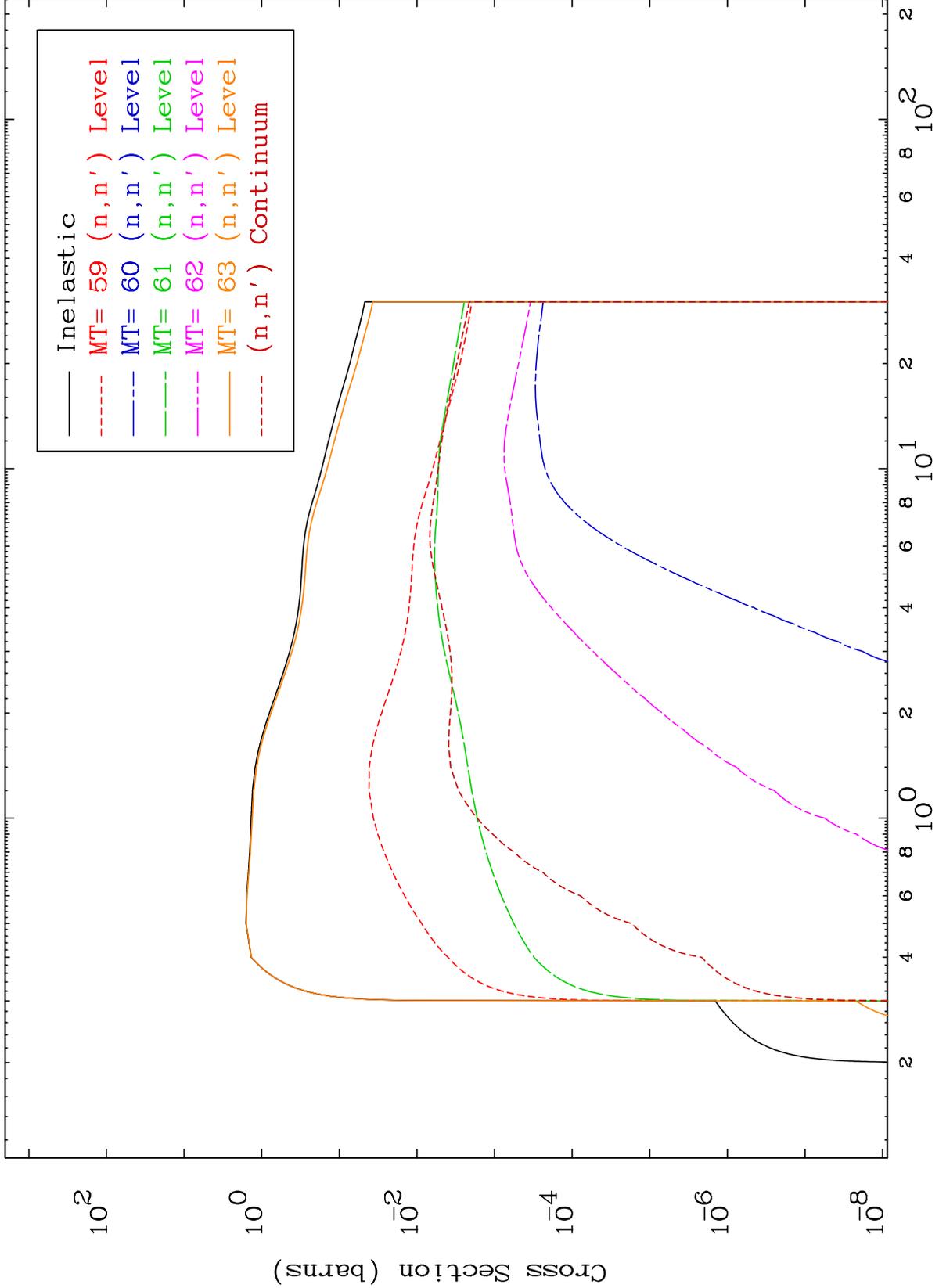
MAT 1182

Fission
293 Kelvin Cross Sections

111-Rg-282



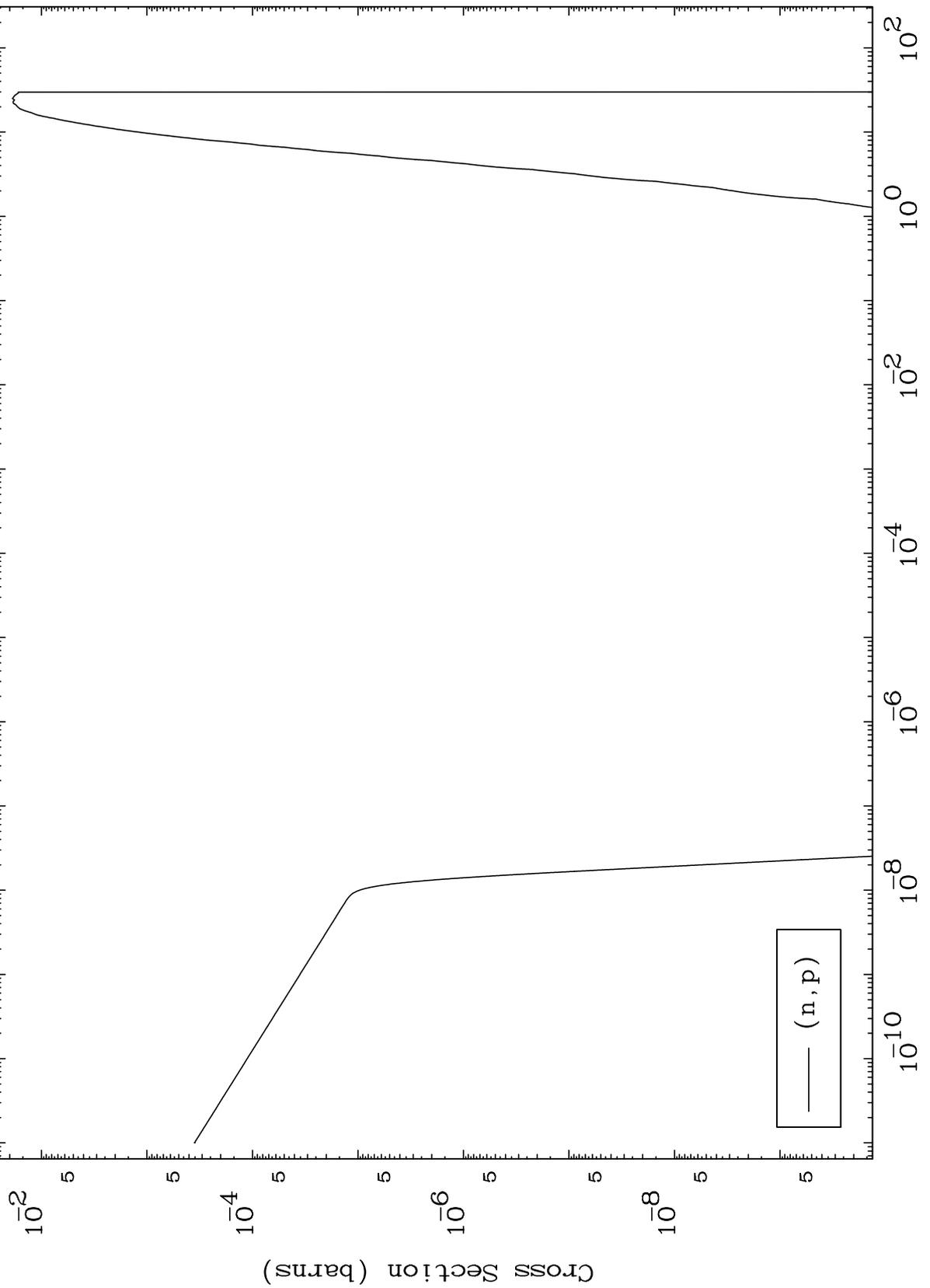




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(n,p) Levels
293 Kelvin Cross Sections

111-Rg-282

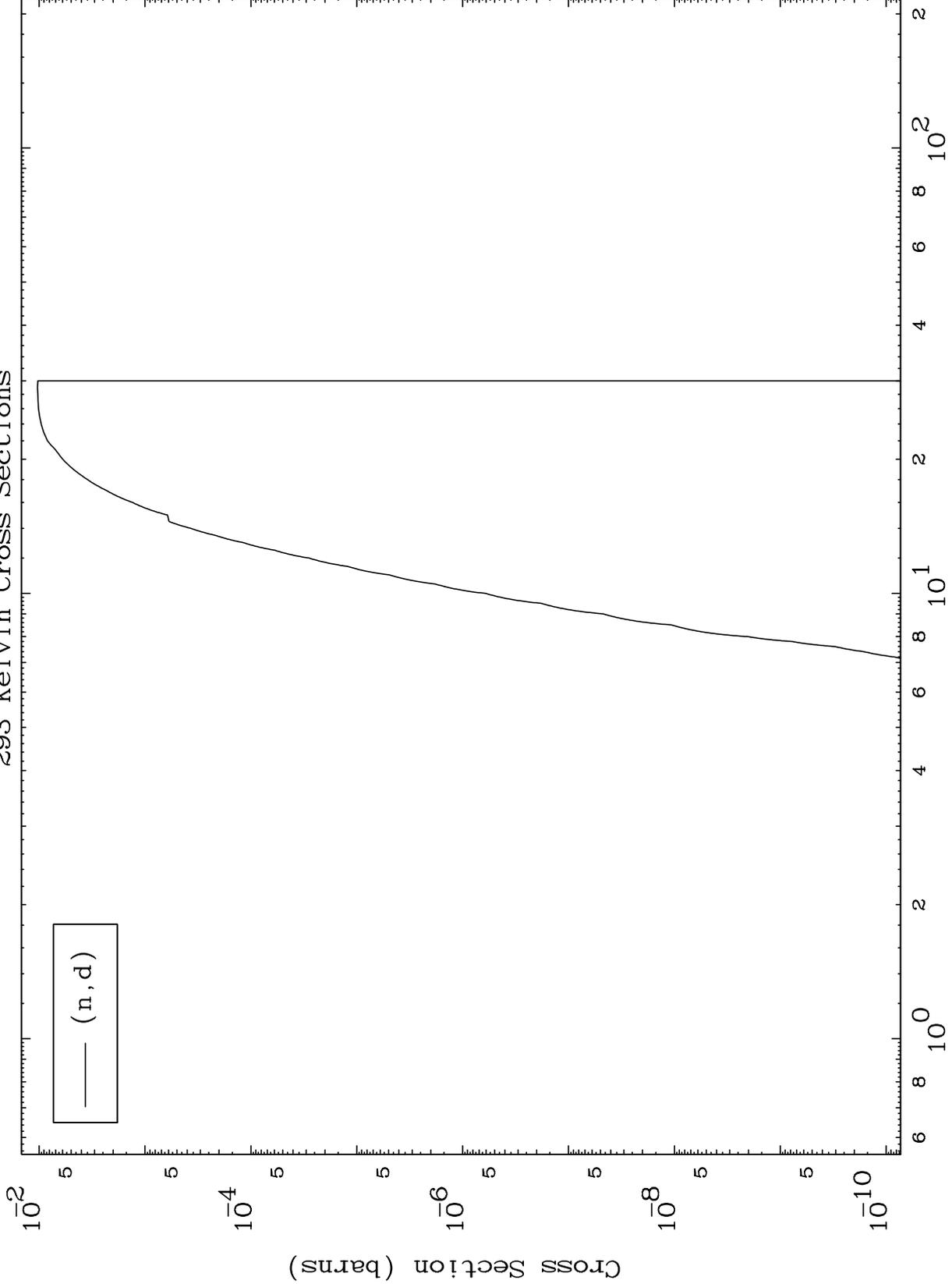


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(n,d) Levels

111-Rg-282

293 Kelvin Cross Sections



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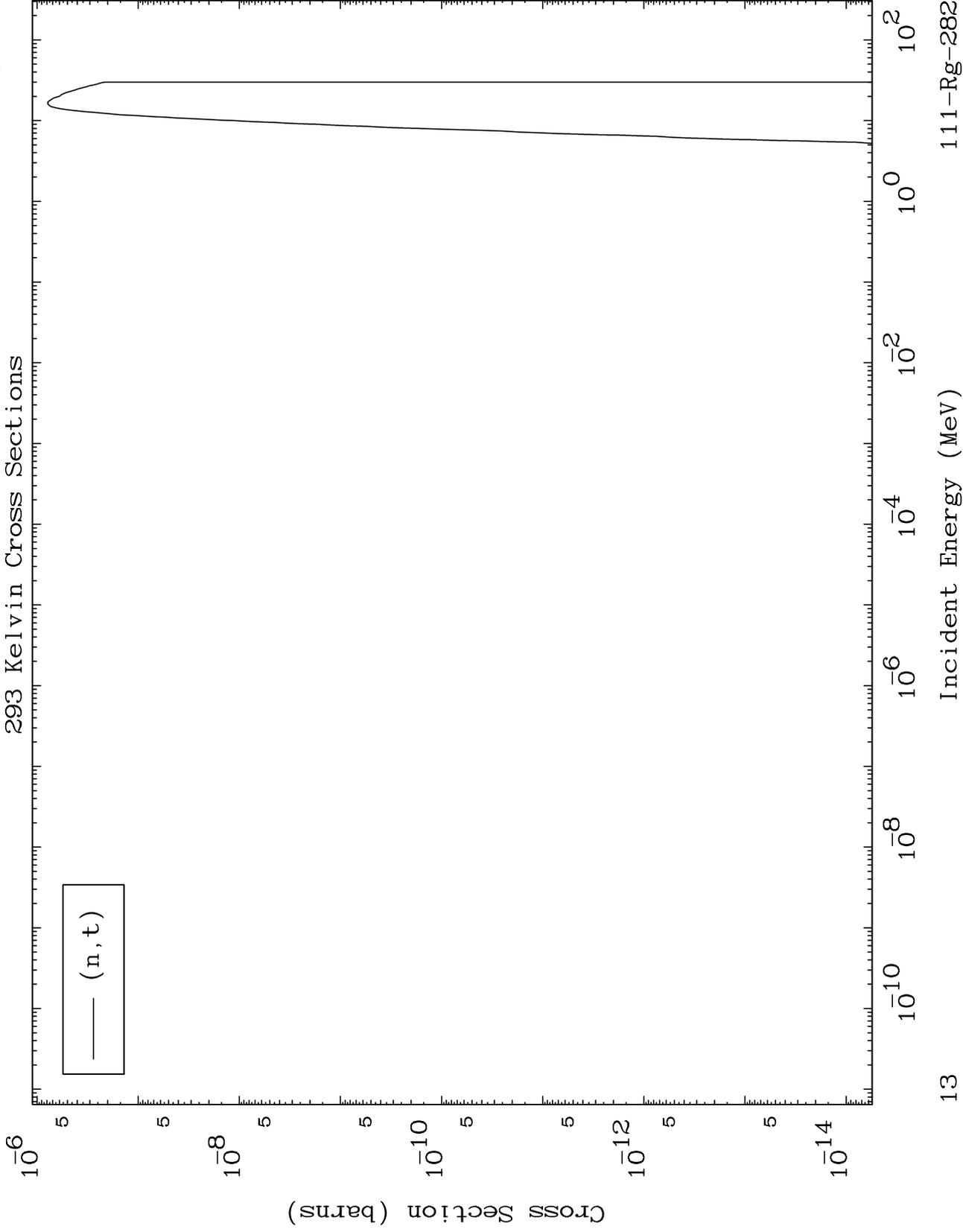
Incident Energy (MeV)

111-Rg-282

MAT 1182

(n,t) Levels
293 Kelvin Cross Sections

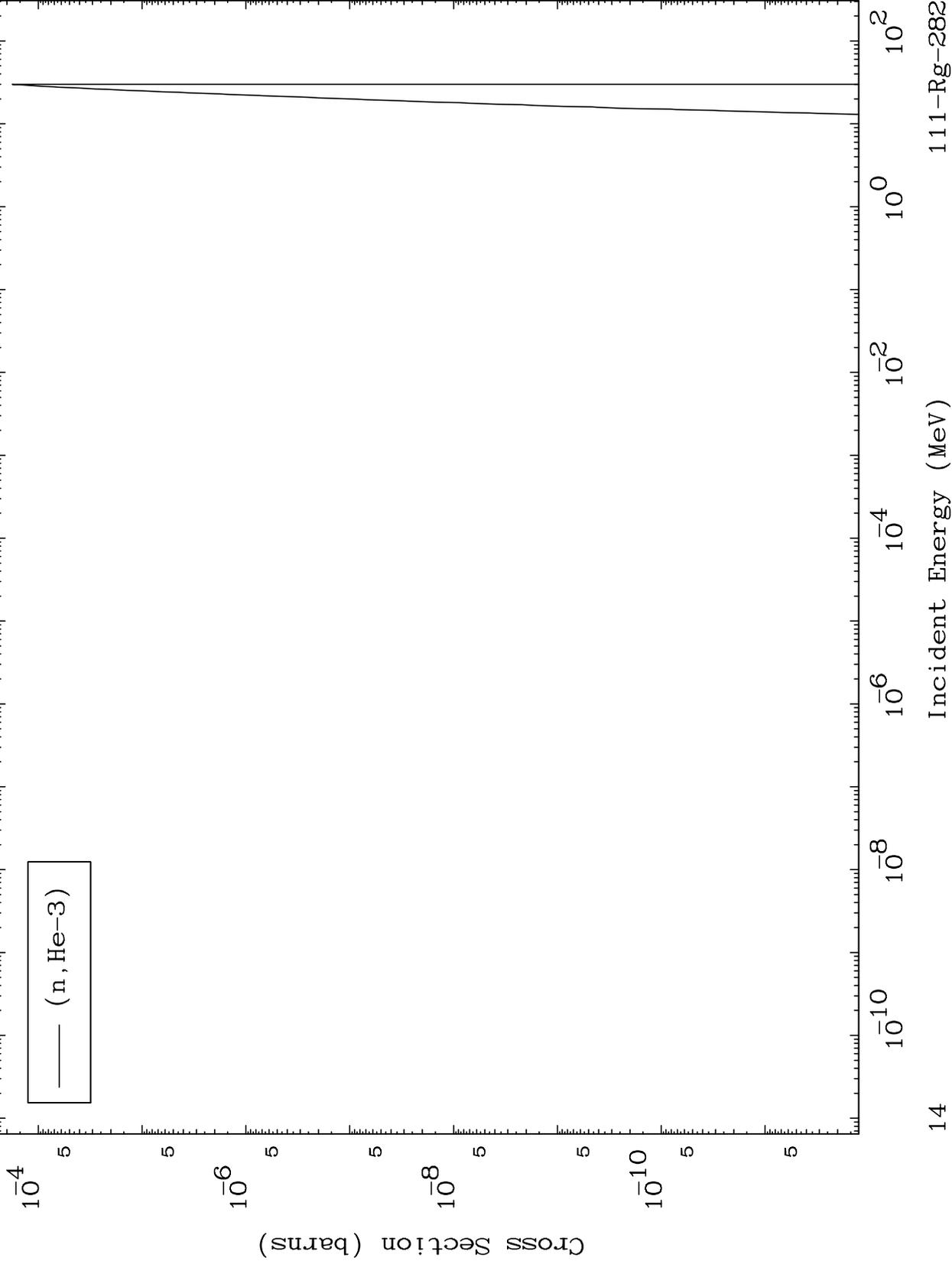
111-Rg-282



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(n,He3) Levels
293 Kelvin Cross Sections

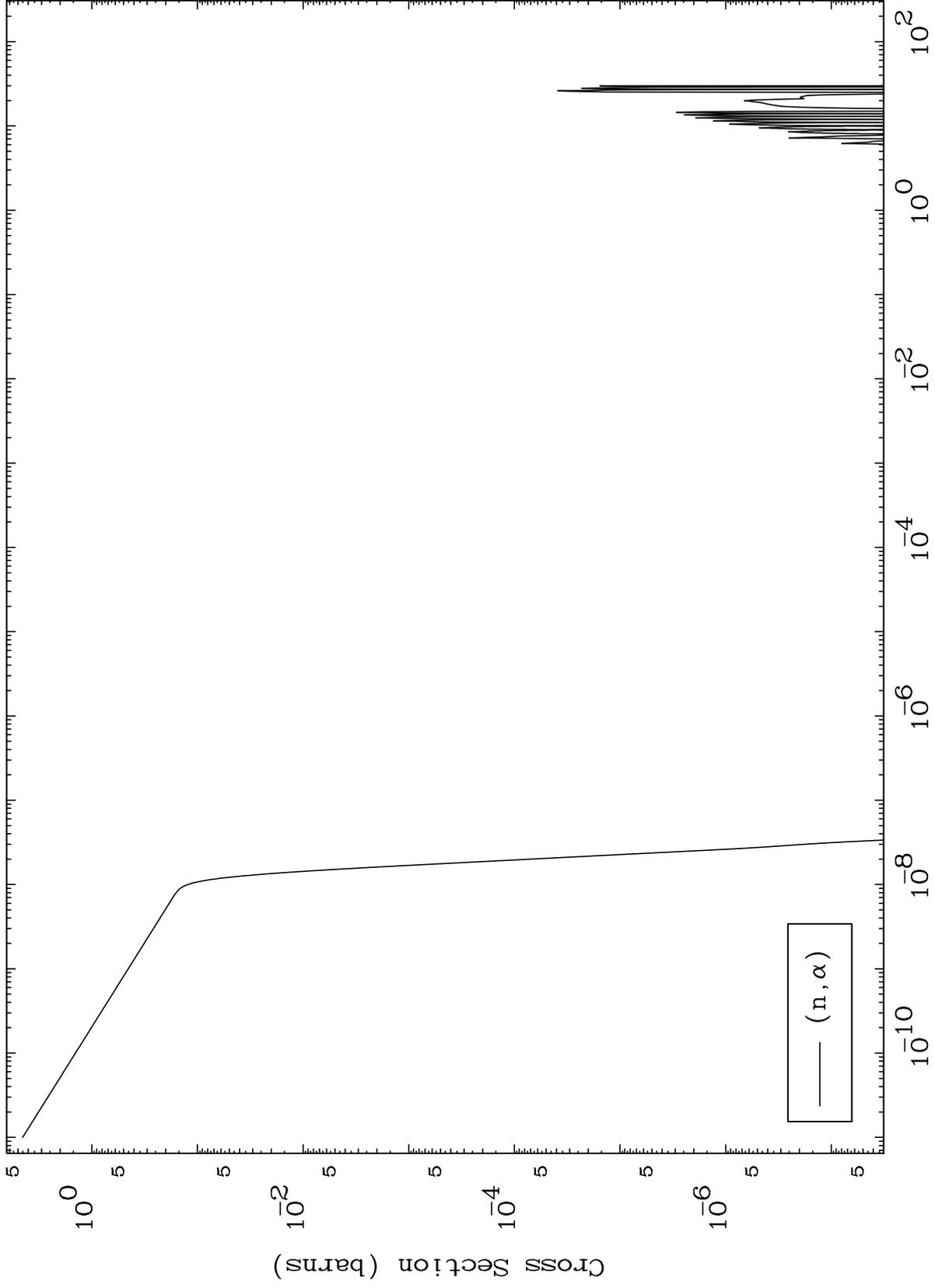
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(n, α) Levels
293 Kelvin Cross Sections

111-Rg-282



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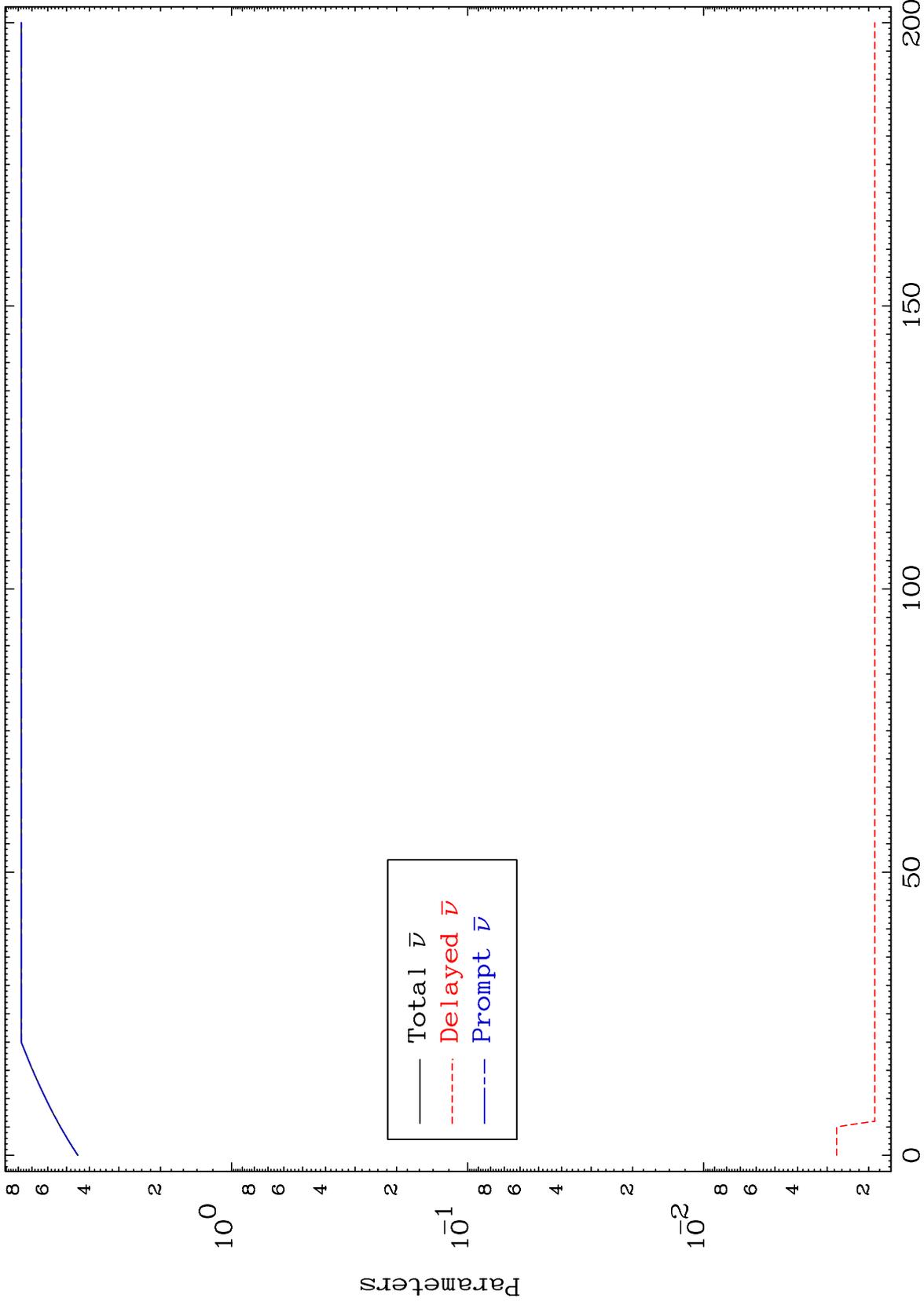
Incident Energy (MeV)

111-Rg-282

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Energy Release
Parameters

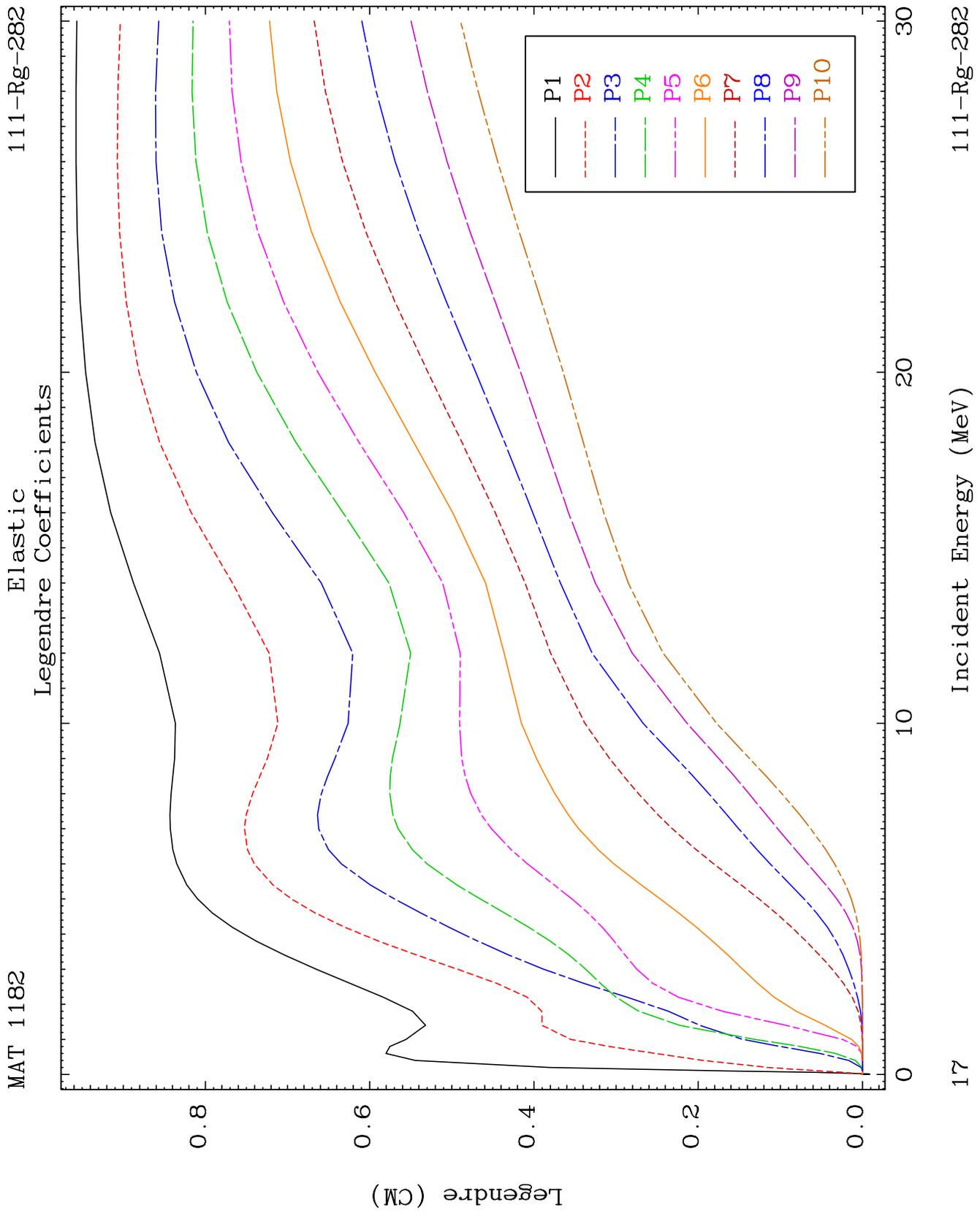
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Incident Energy (MeV)

111-Rg-282



111-Rg-282

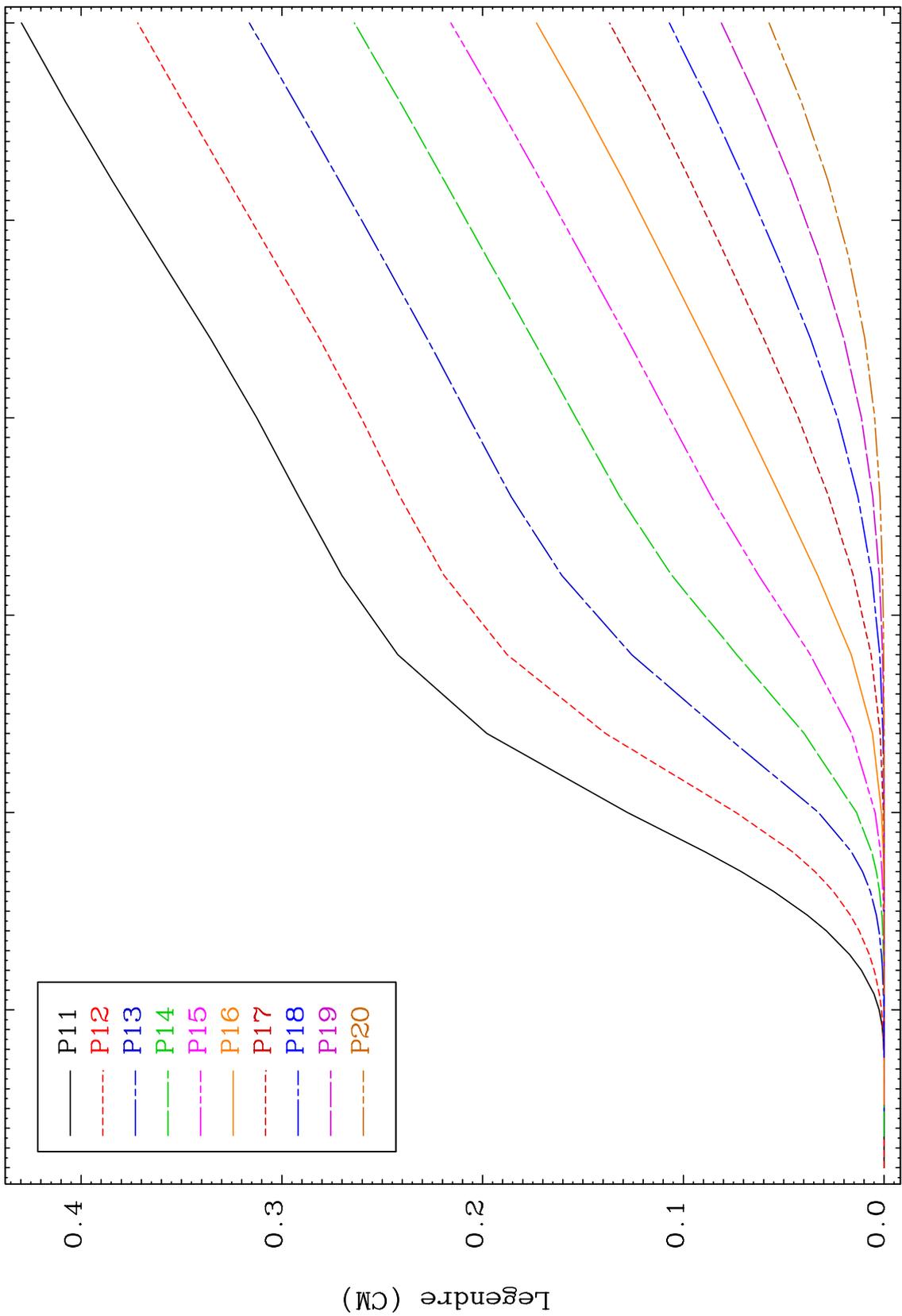
Incident Energy (MeV)

17

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Elastic Legendre Coefficients

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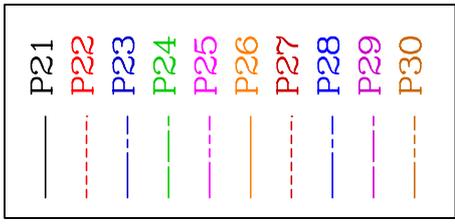
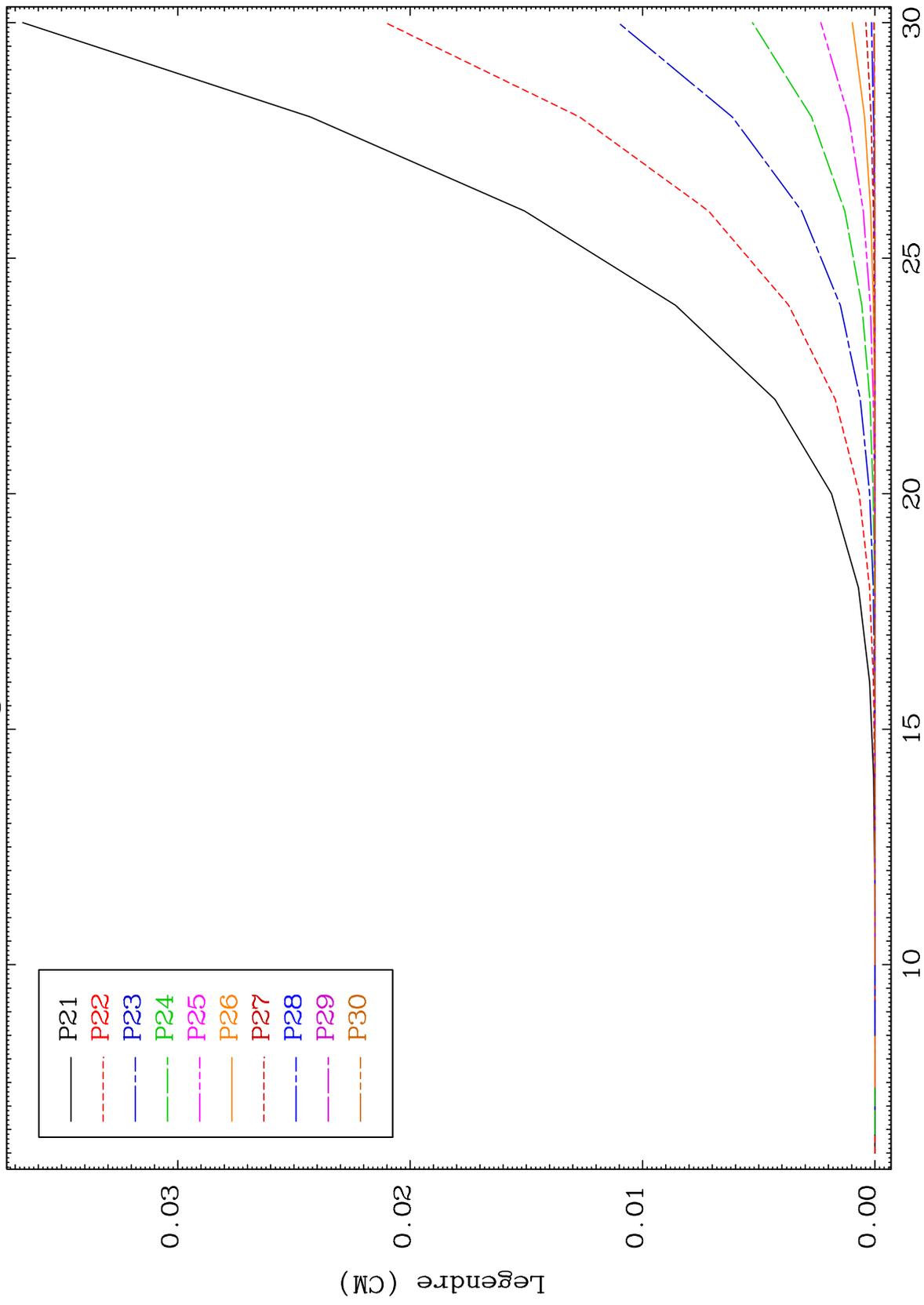
Incident Energy (MeV)

111-Rg-282

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Elastic
Legendre Coefficients

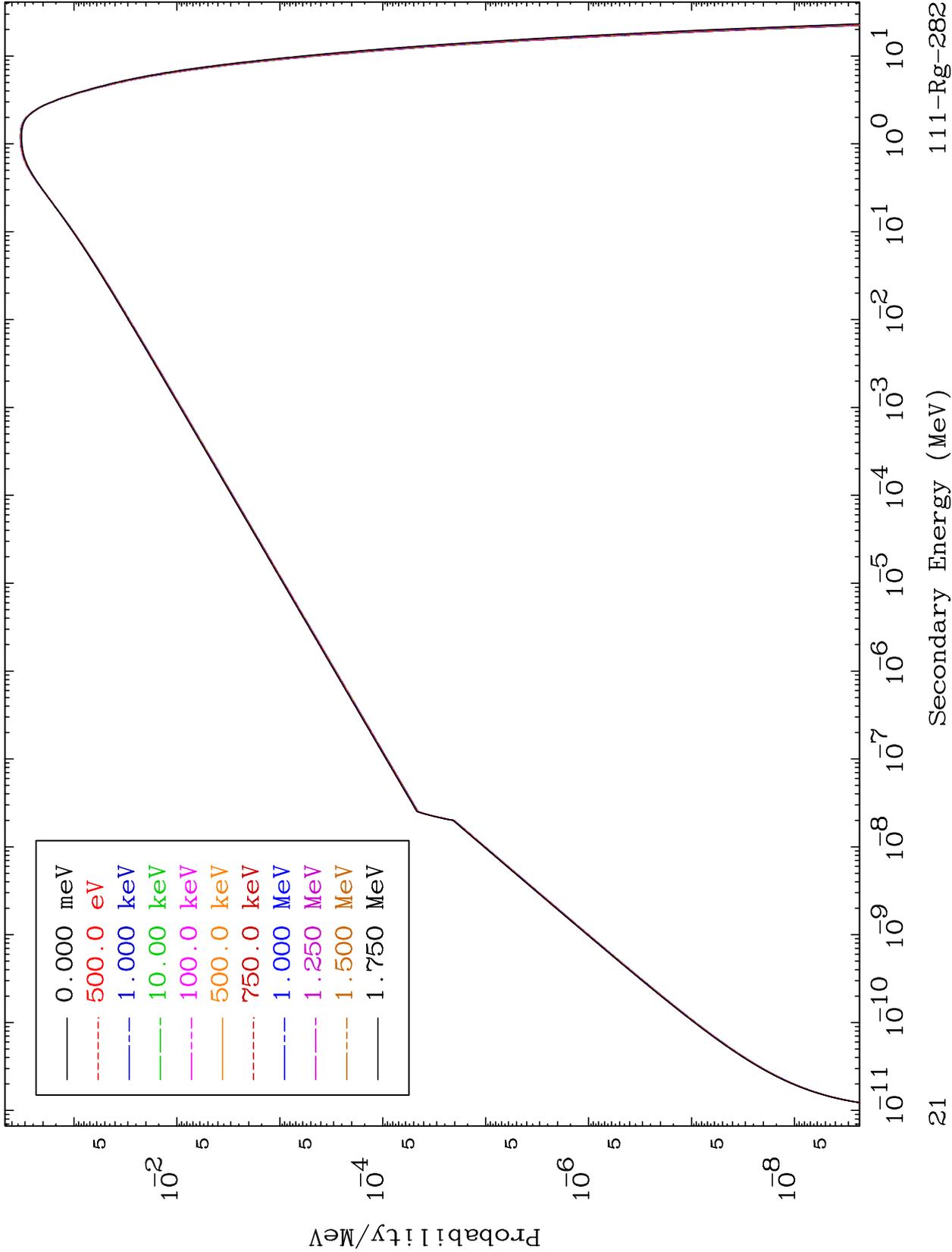
111-Rg-282



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Fission Energy Distributions

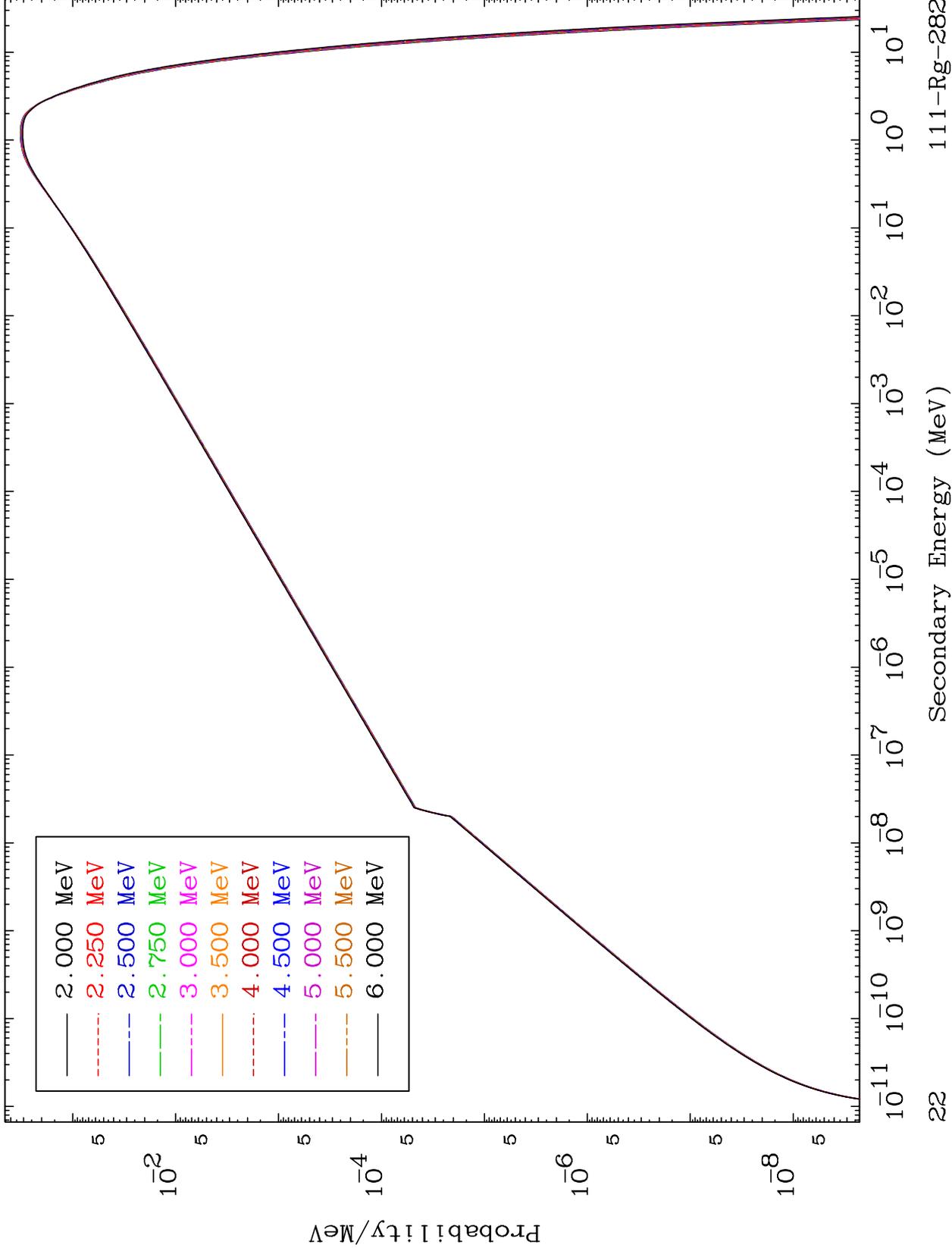
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Fission Energy Distributions

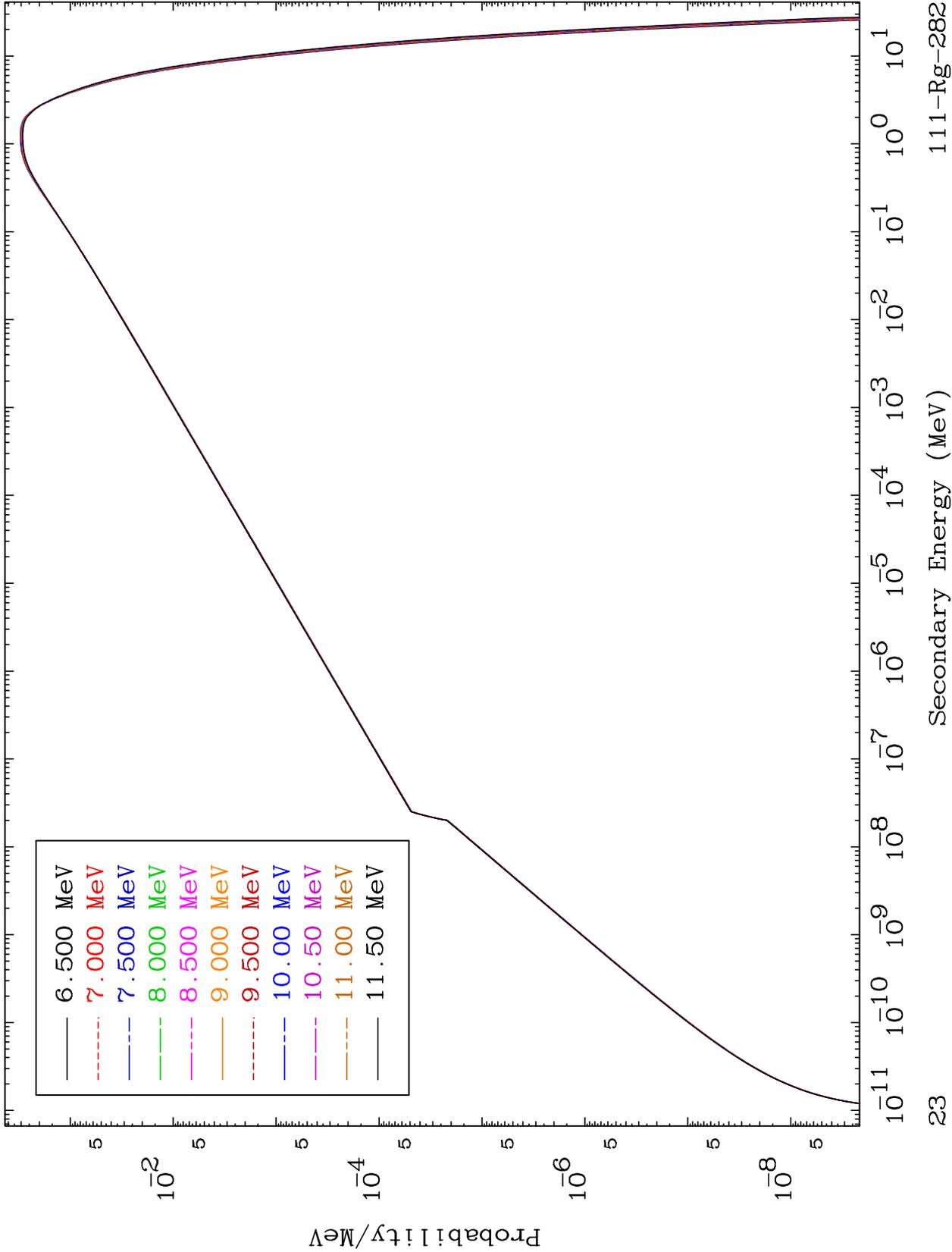
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Fission Energy Distributions

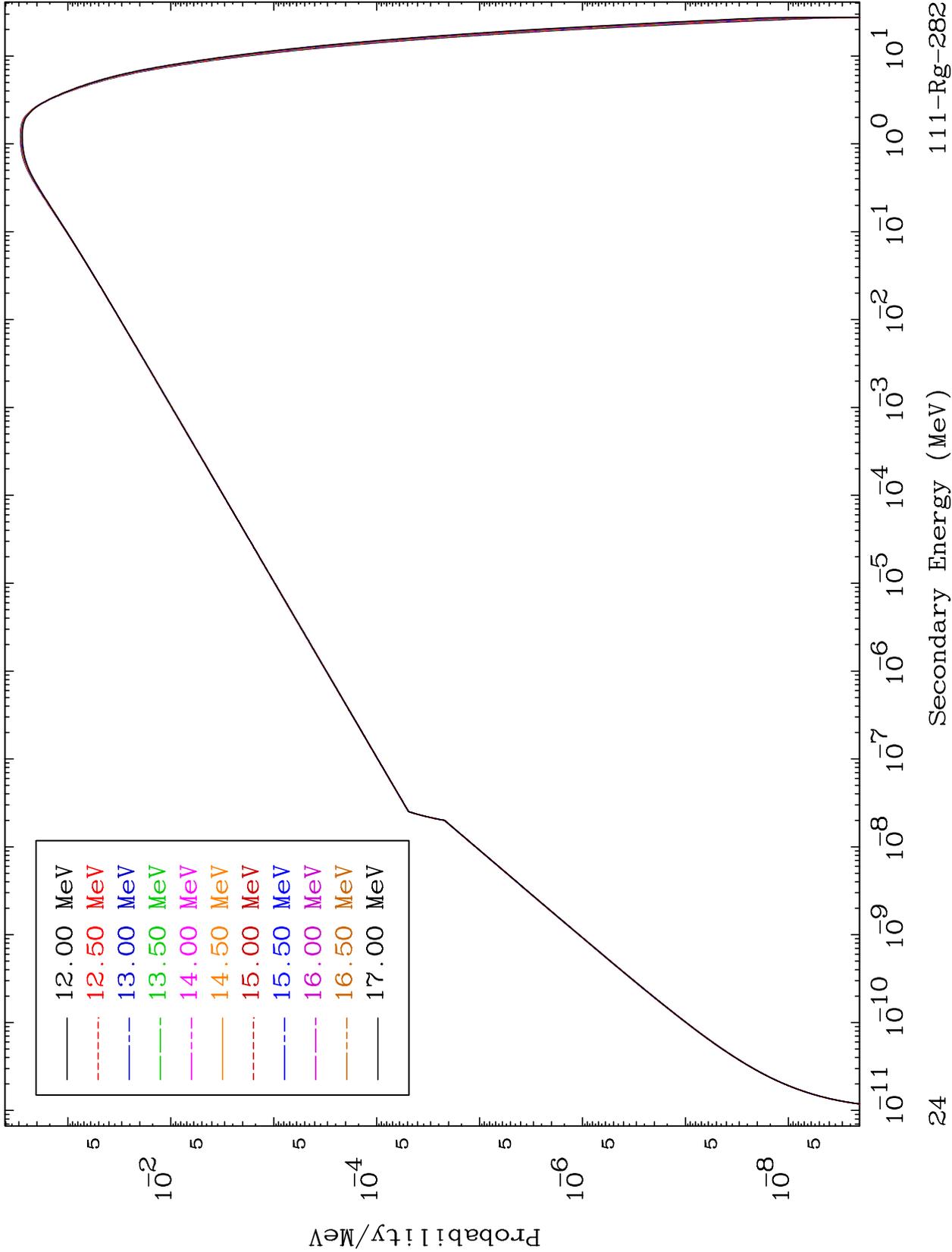
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Fission Energy Distributions

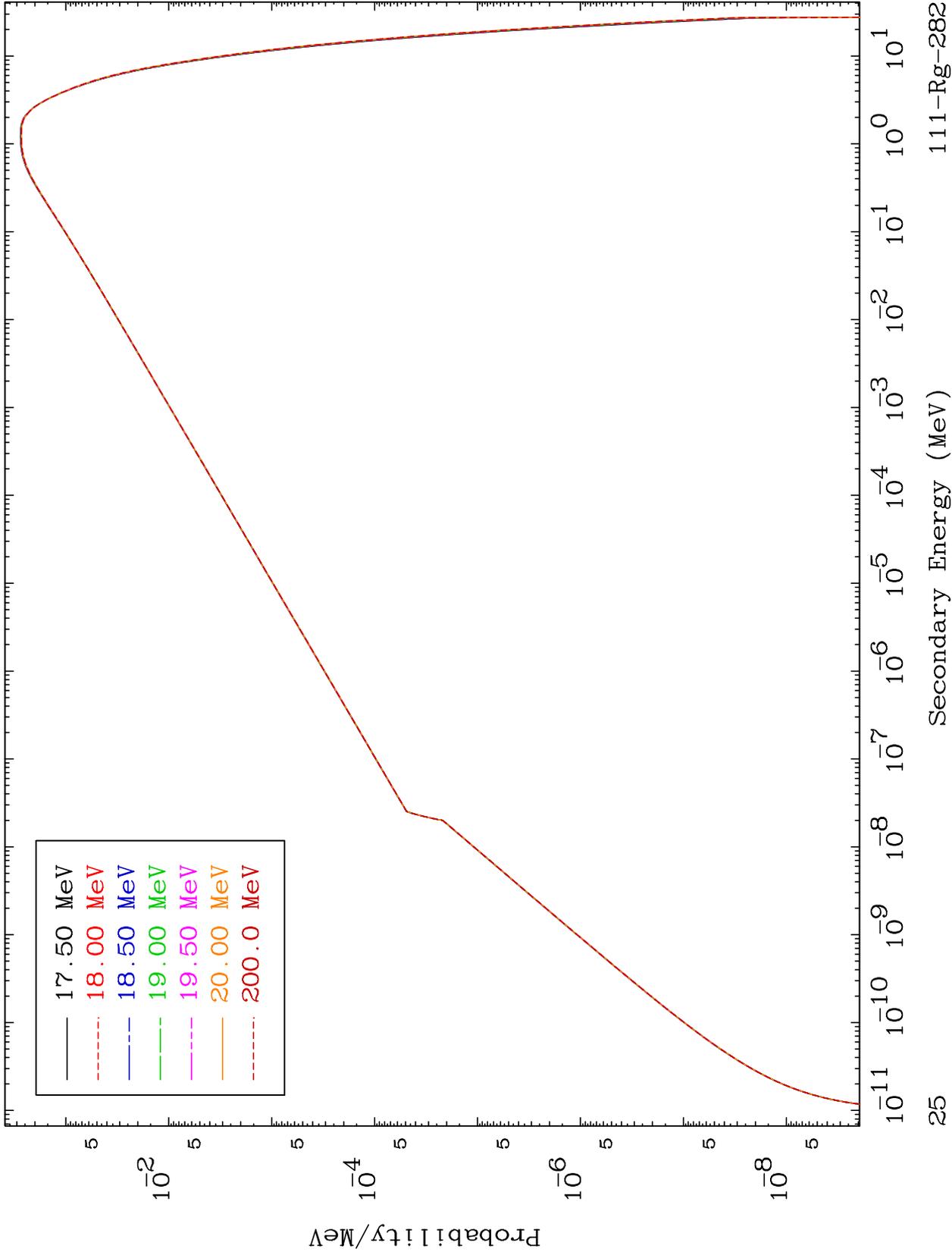
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Secondary Energy (MeV)

Fission Energy Distributions

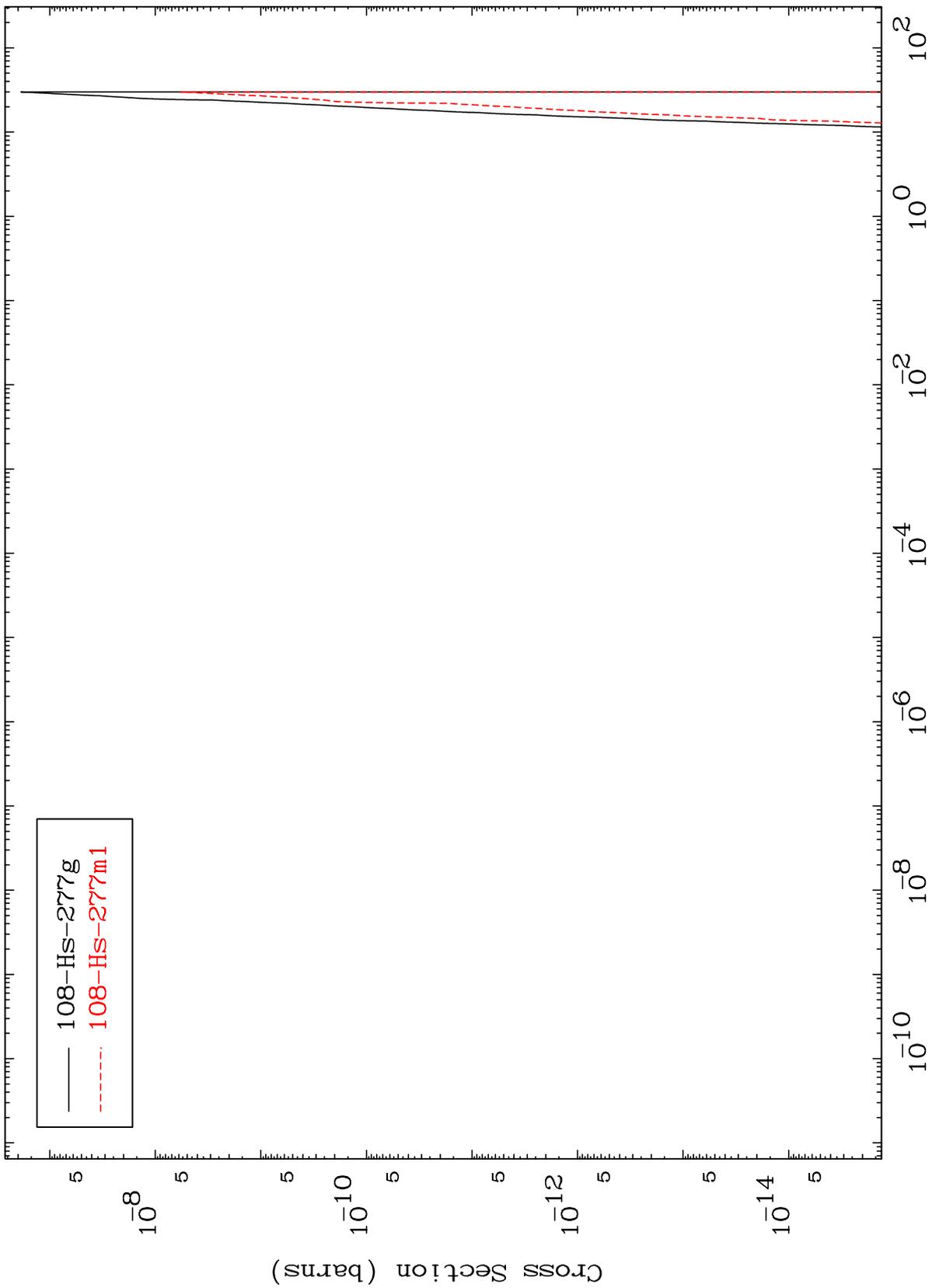


MAT 1182

(n,n') p α

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Radionuclide Production Cross Section



MAT 1182

(n,d) α

111-Rg-282

Radionuclide Production Cross Section

