

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

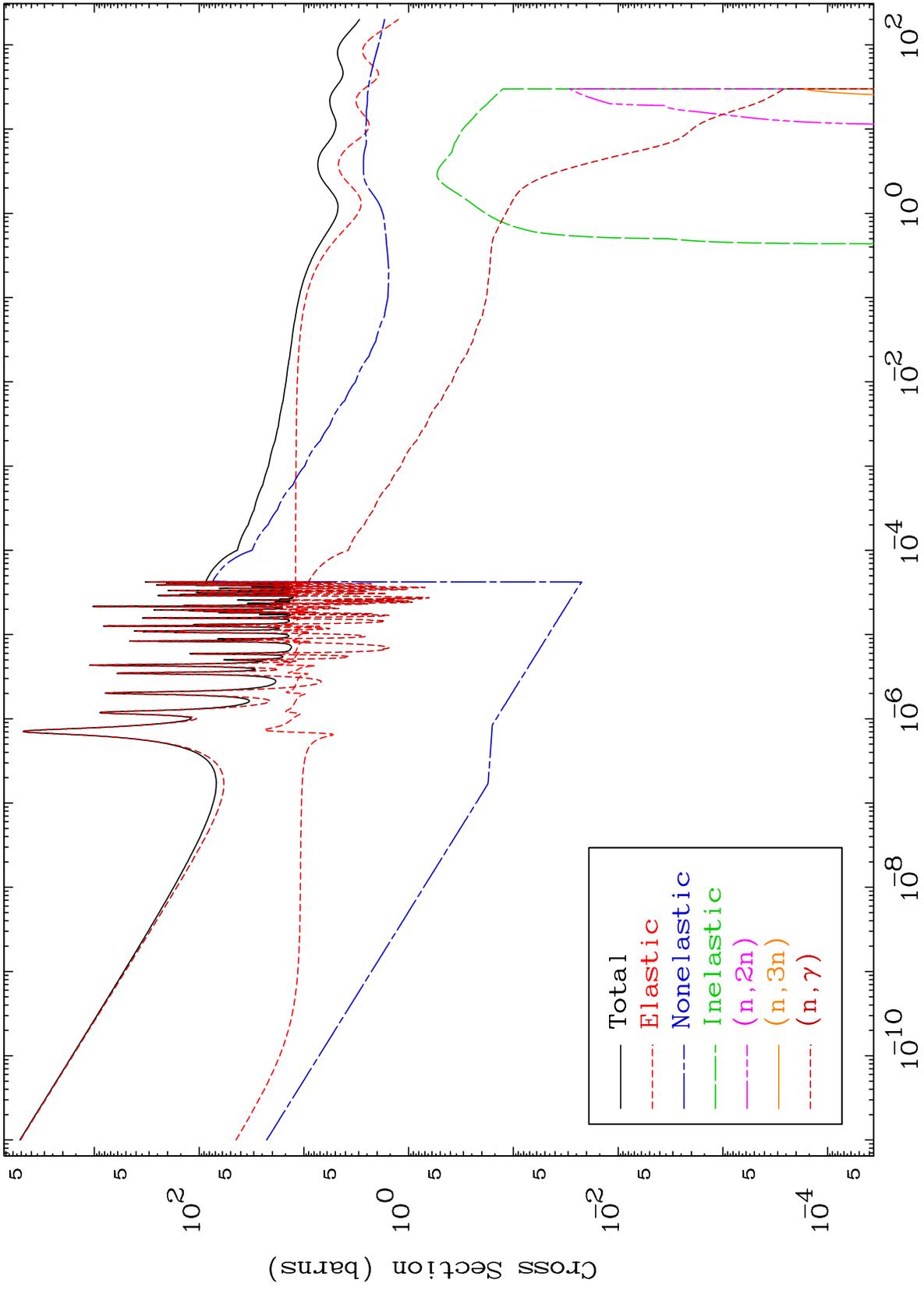
E.Mail:redcullen1@comcast.net
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 8592

Neutron Major
293 Kelvin Cross Sections

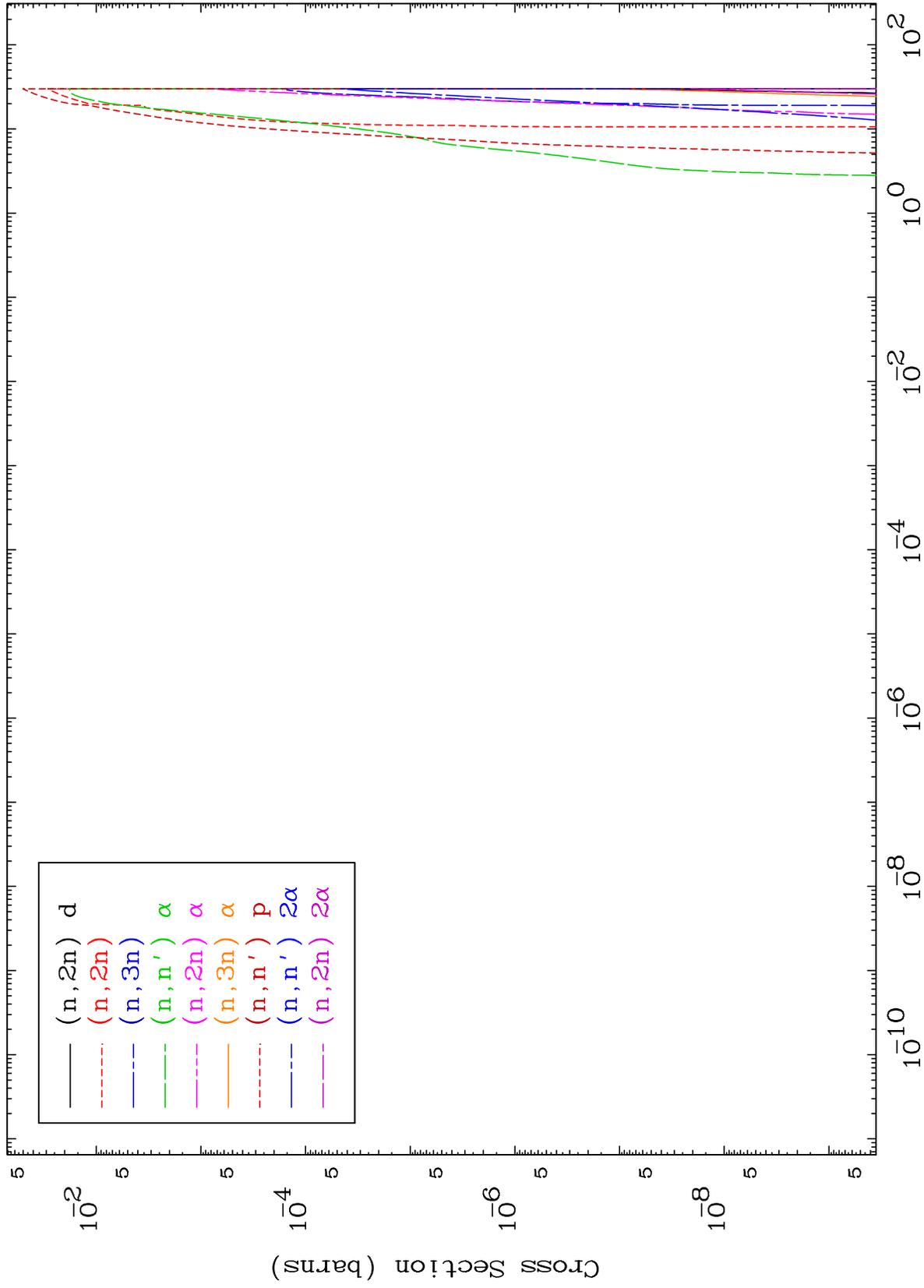
86-Rn-200



MAT 8592

Neutron Absorption
293 Kelvin Cross Sections

86-Rn-200

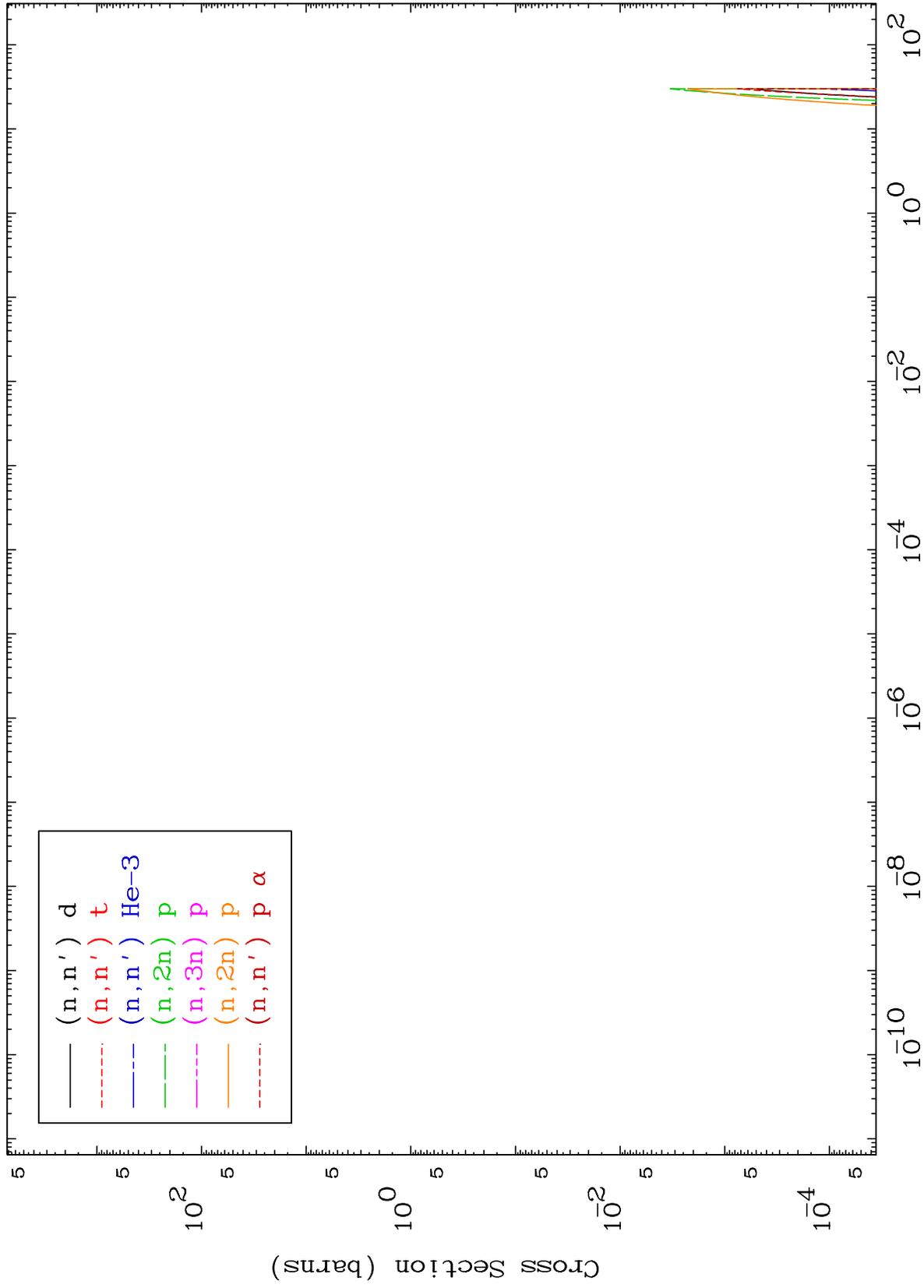


86-Rn-200

MAT 8592

Neutron Absorption
293 Kelvin Cross Sections

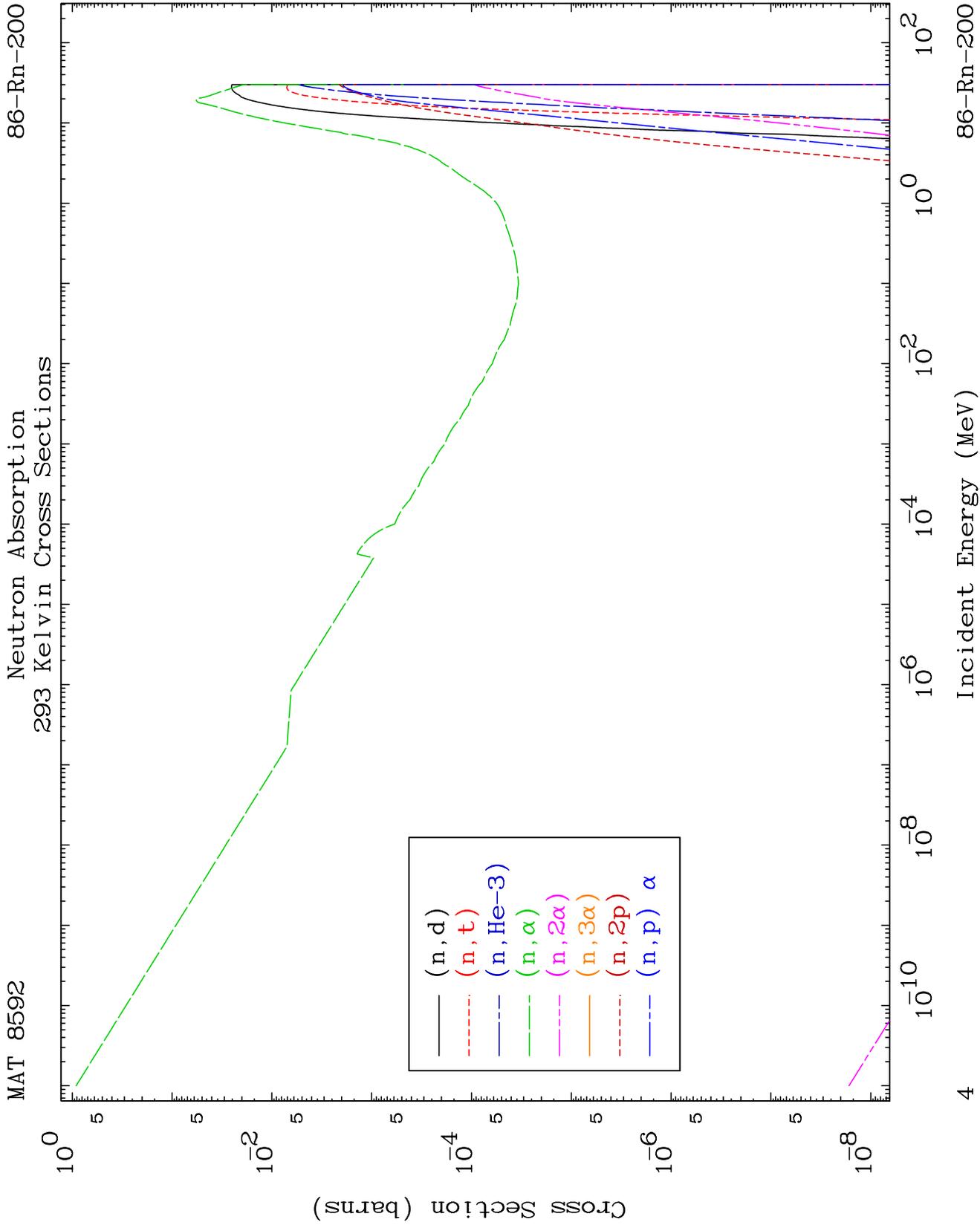
86-Rn-200



3

Incident Energy (MeV)

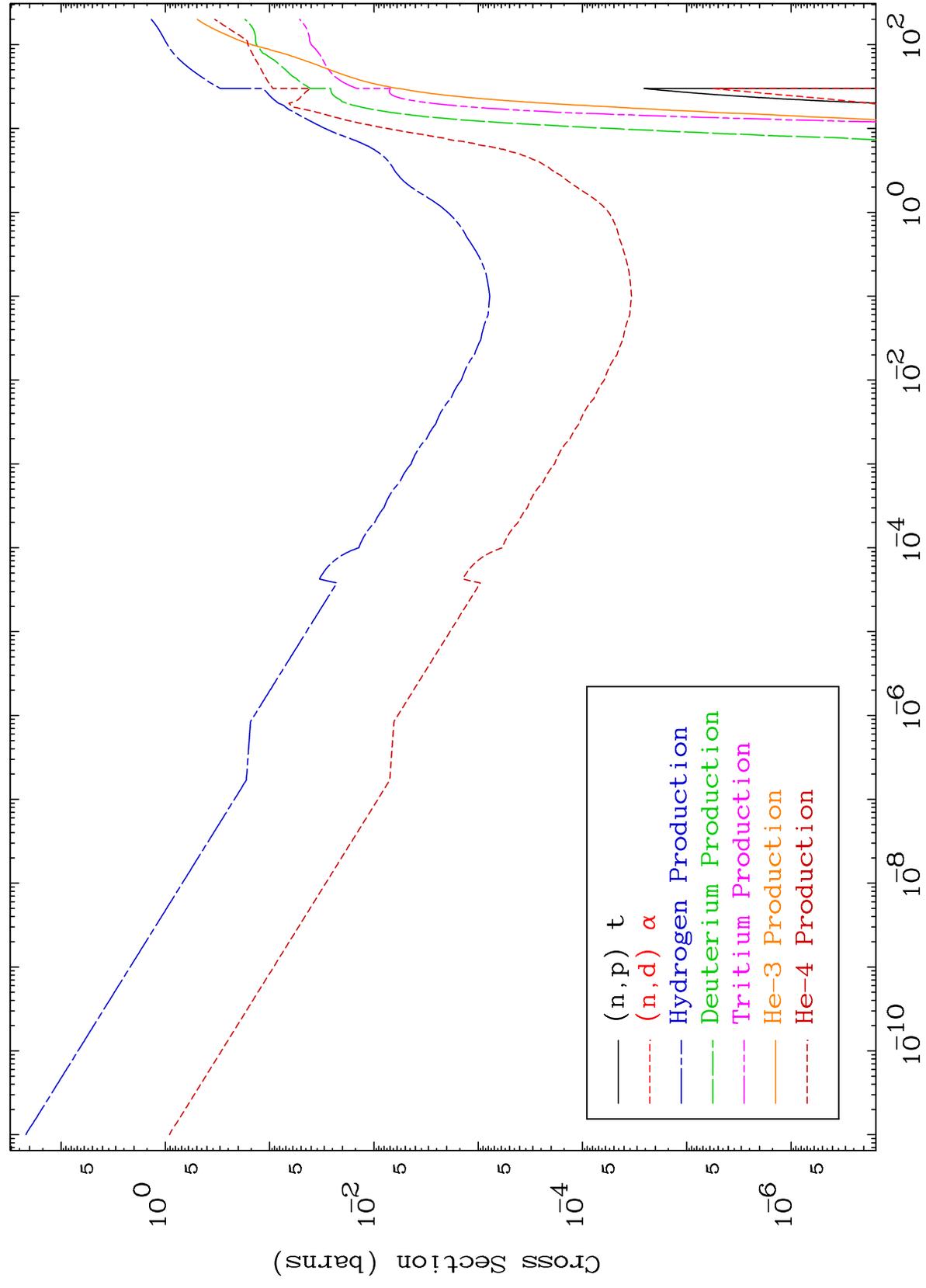
86-Rn-200



MAT 8592

Neutron Absorption
293 Kelvin Cross Sections

86-Rn-200

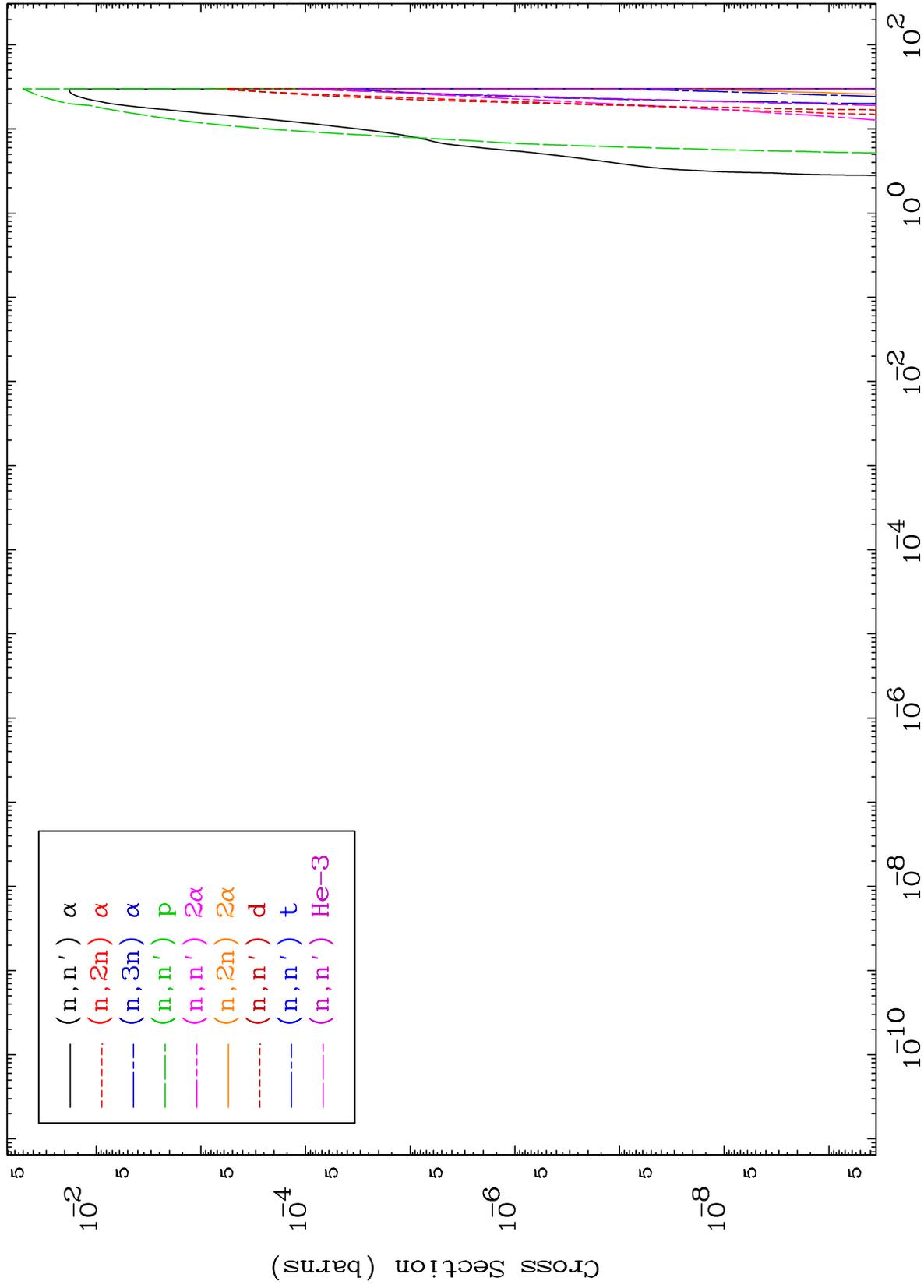


86-Rn-200

MAT 8592

293 Kelvin Cross Sections

86-Rn-200



6

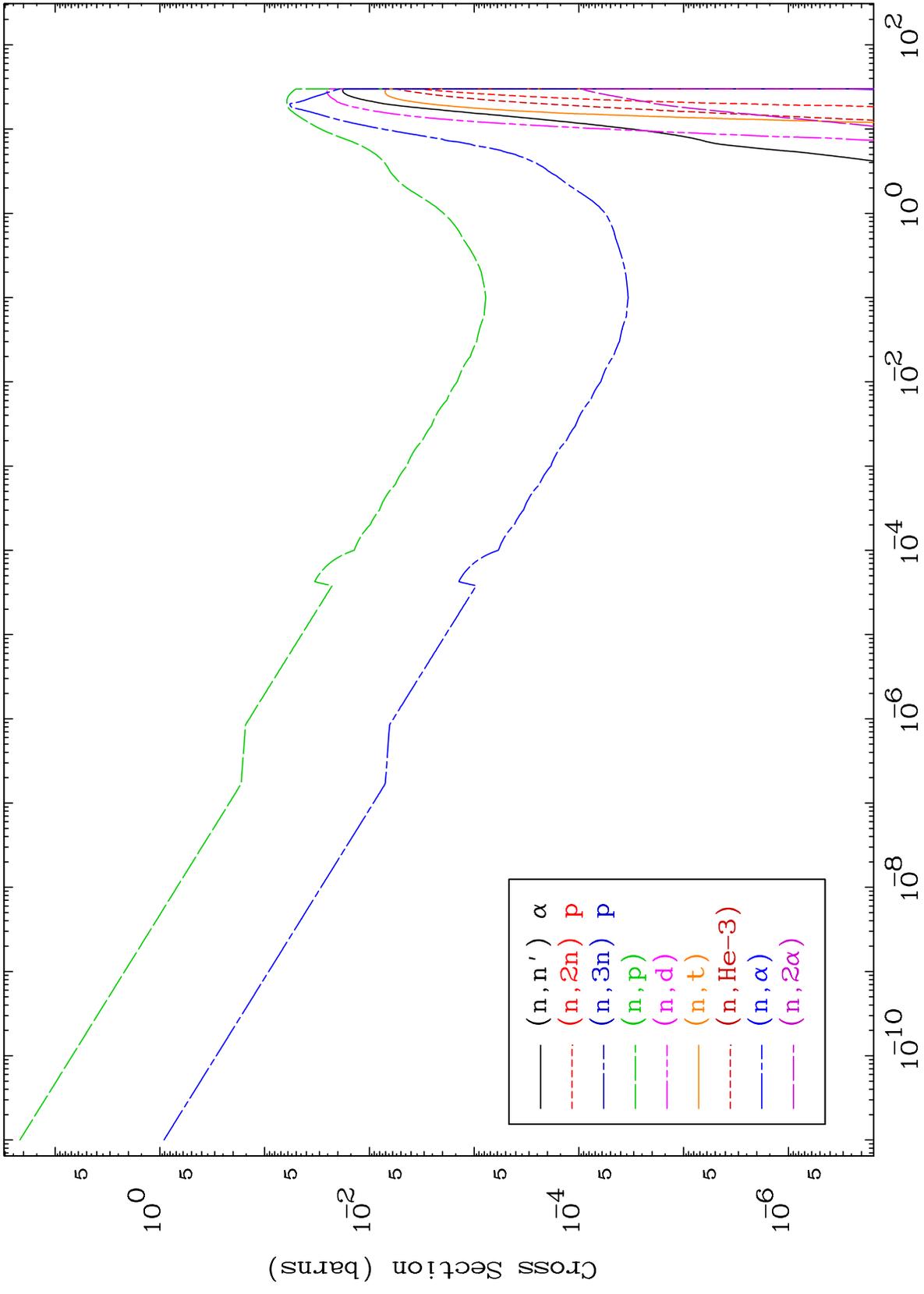
Incident Energy (MeV)

86-Rn-200

MAT 8592

293 Kelvin Cross Sections

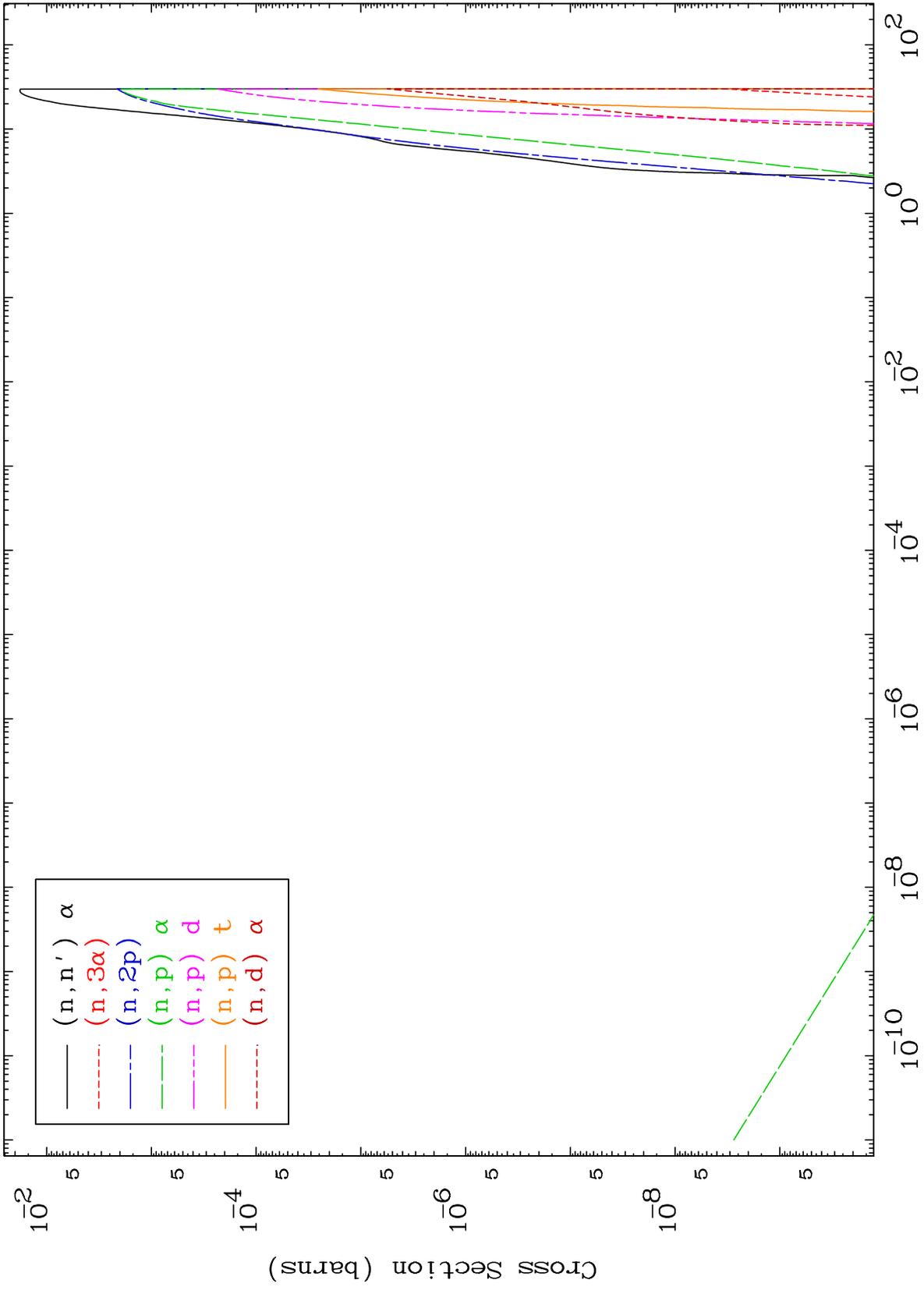
86-Rn-200



MAT 8592

293 Kelvin Cross Sections

86-Rn-200

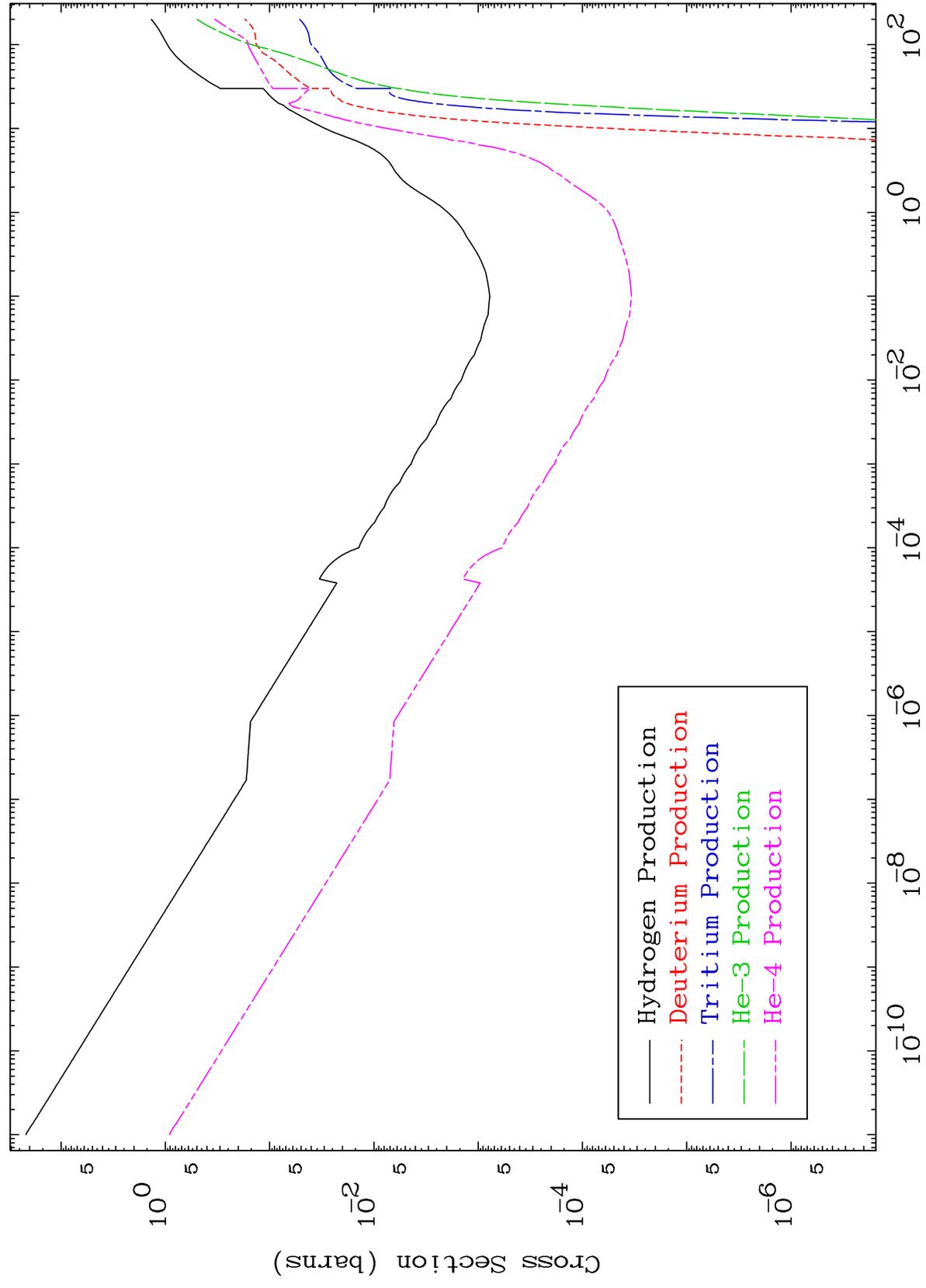


86-Rn-200

MAT 8592

293 Kelvin Cross Sections

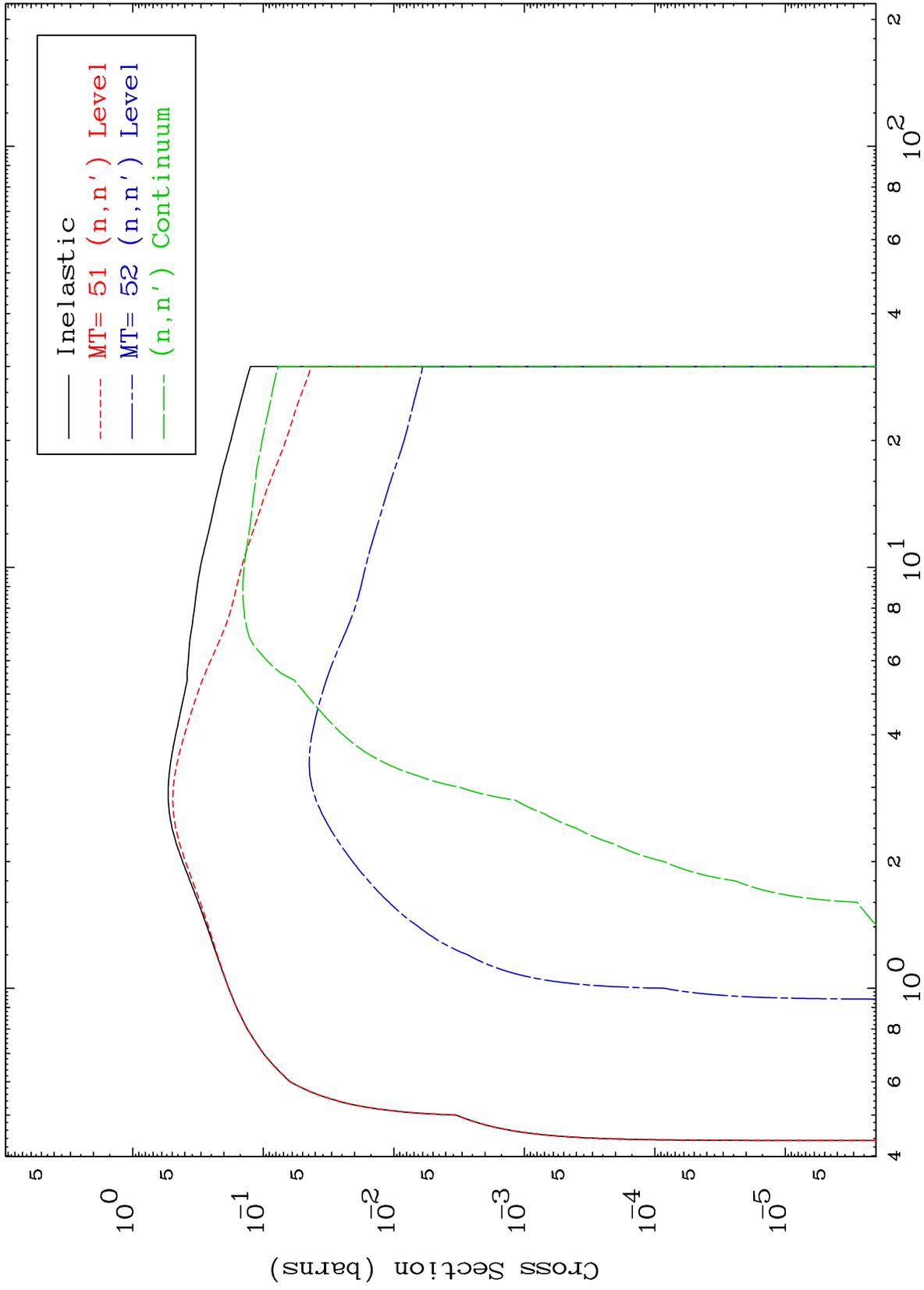
86-Rn-200



MAT 8592

(n,n') Levels
293 Kelvin Cross Sections

86-Rn-200



10

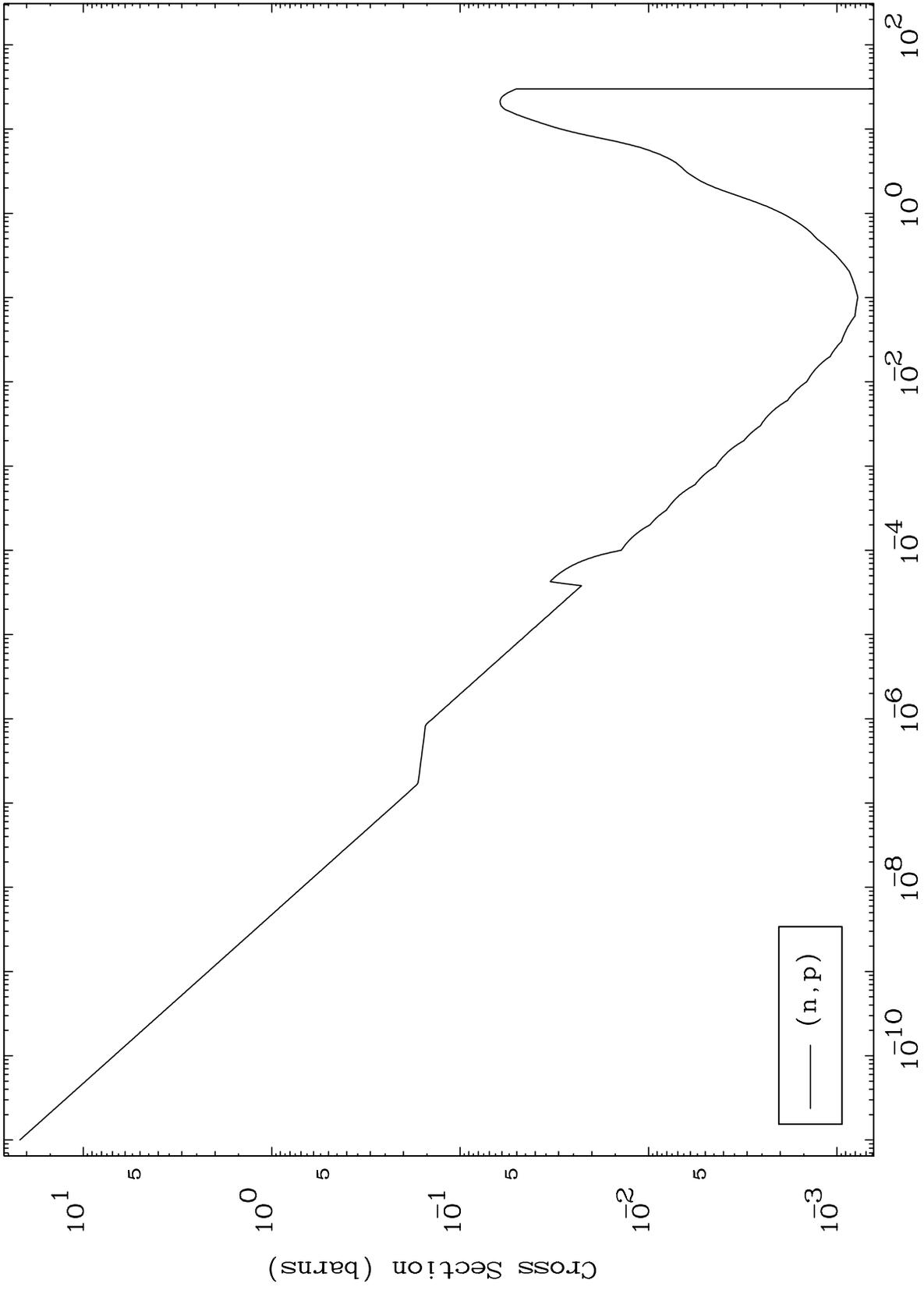
Incident Energy (MeV)

86-Rn-200

MAT 8592

(n,p) Levels
293 Kelvin Cross Sections

86-Rn-200

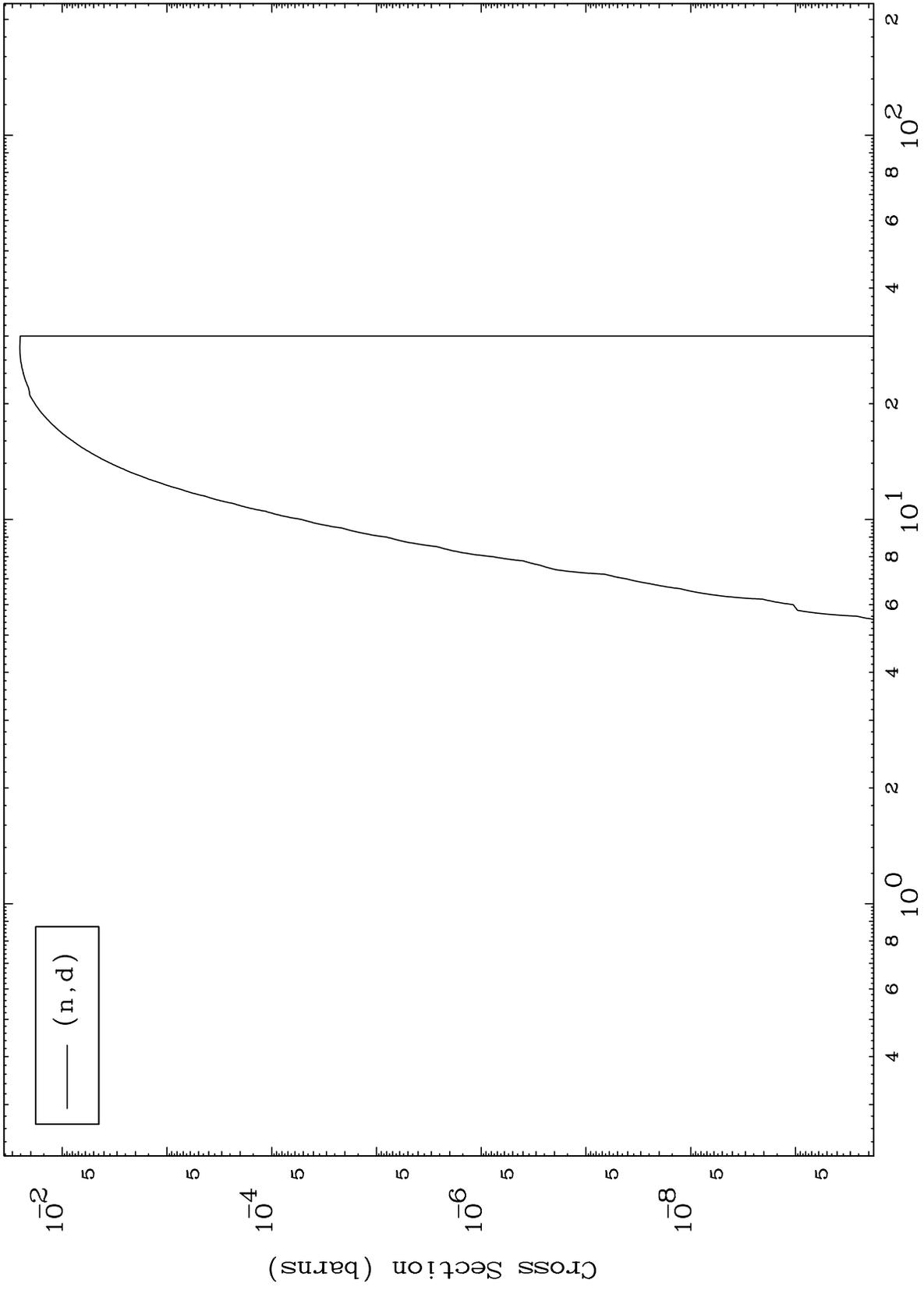


86-Rn-200

MAT 8592

(n,d) Levels
293 Kelvin Cross Sections

86-Rn-200



12

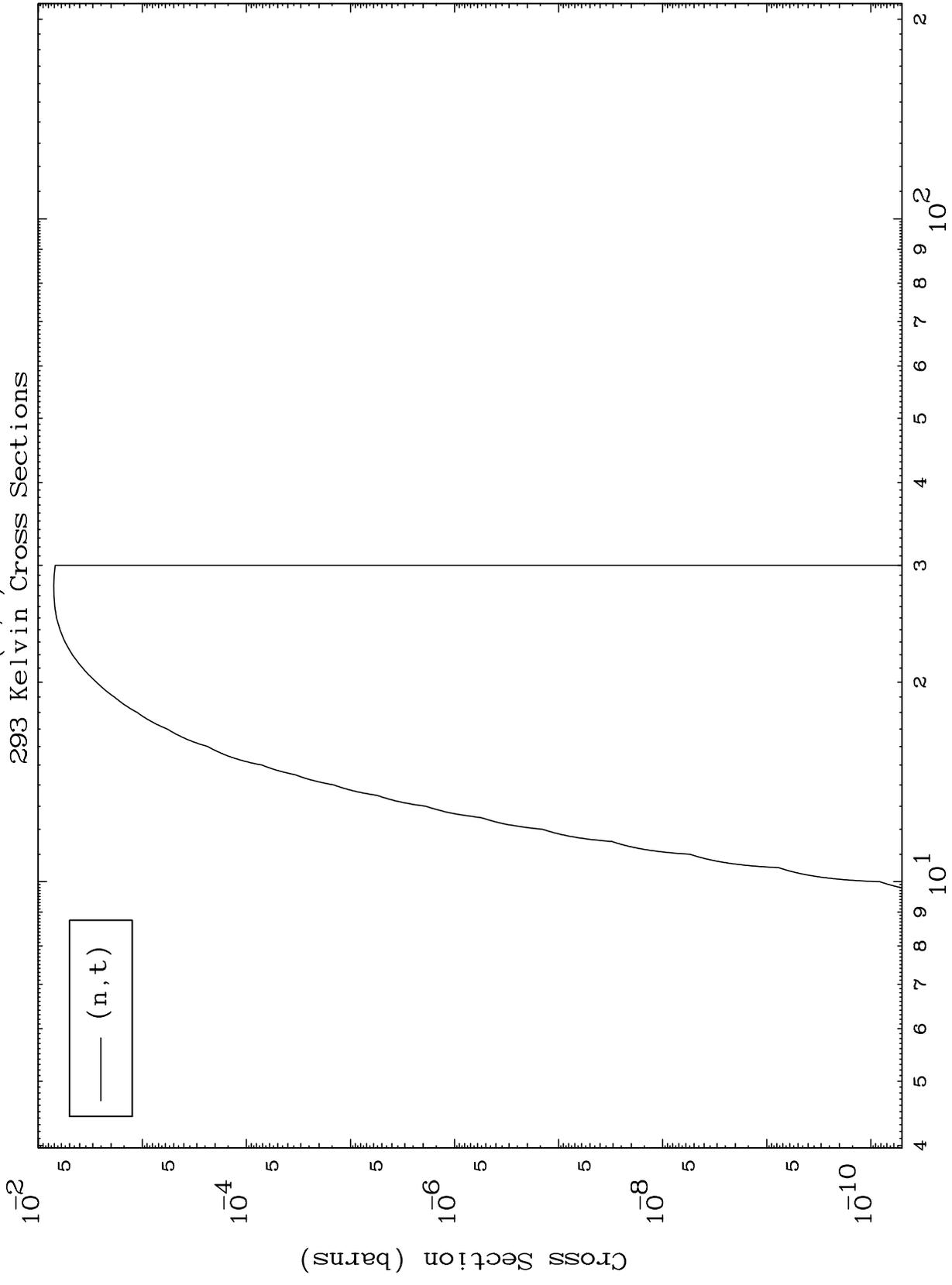
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86-Rn-200

MAT 8592

(n,t) Levels
293 Kelvin Cross Sections

86-Rn-200



13

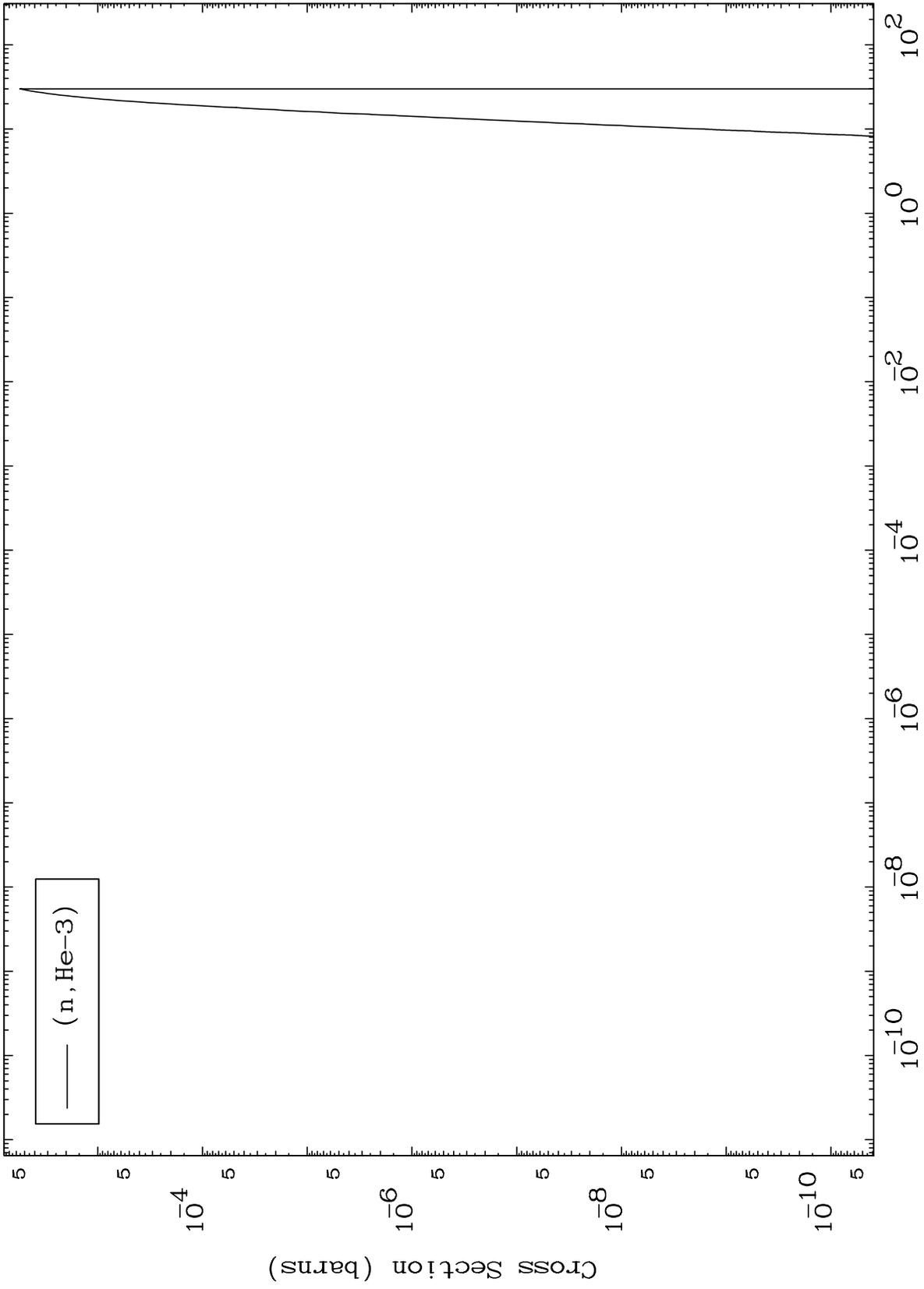
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86-Rn-200

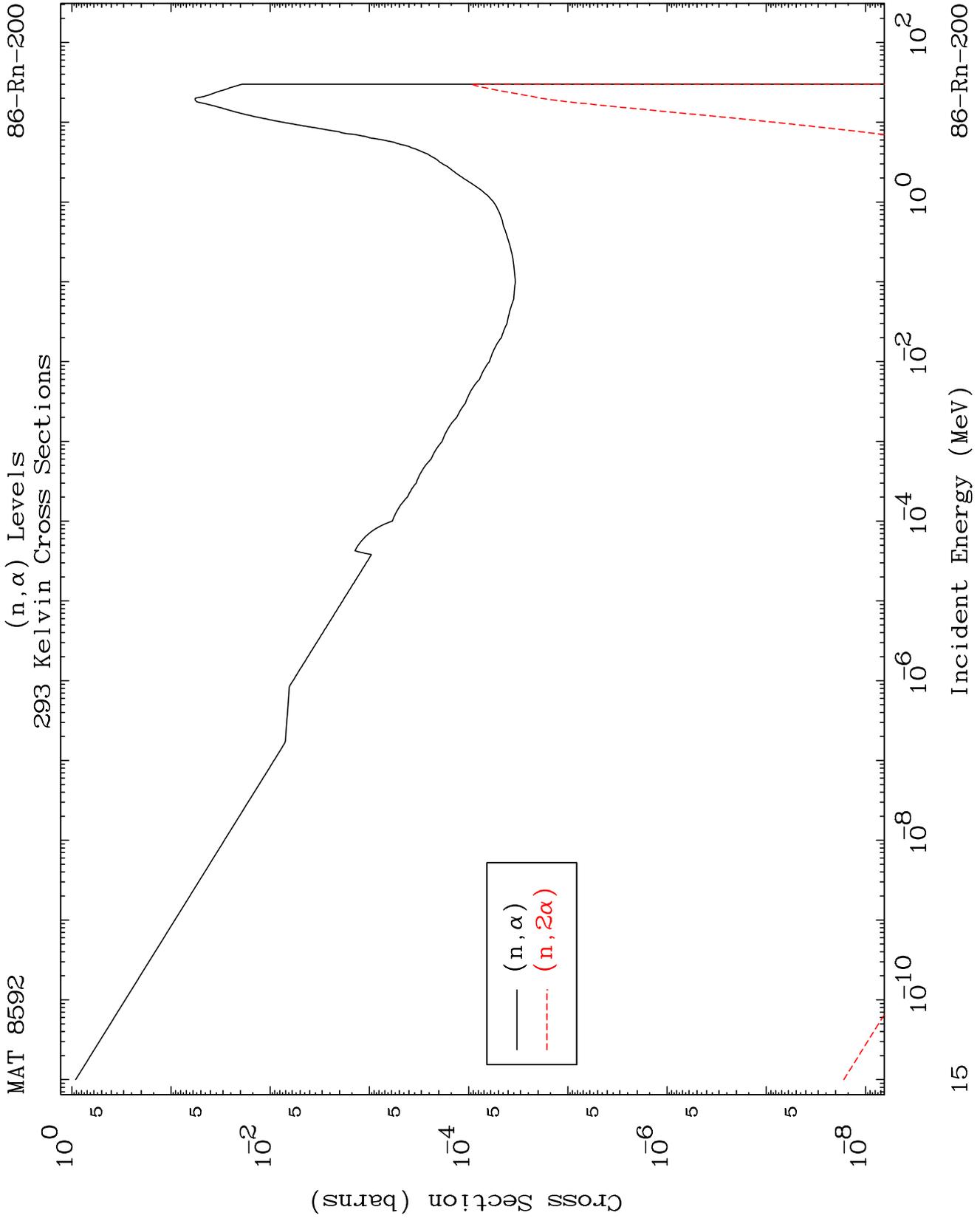
MAT 8592

(n,He3) Levels
293 Kelvin Cross Sections

86-Rn-200



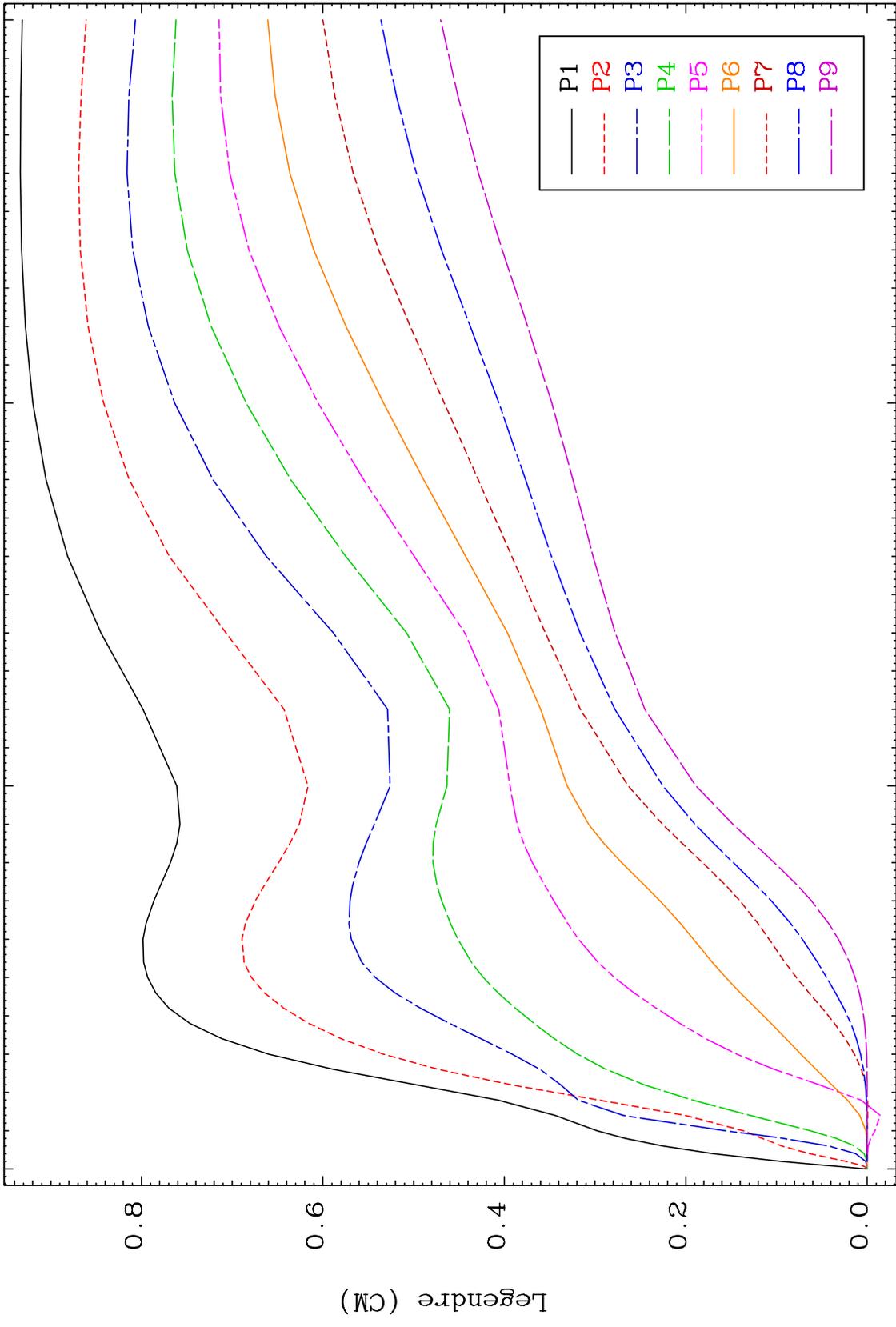
(n, He-3)



MAT 8592

86-Rn-200

Elastic Legendre Coefficients



86-Rn-200

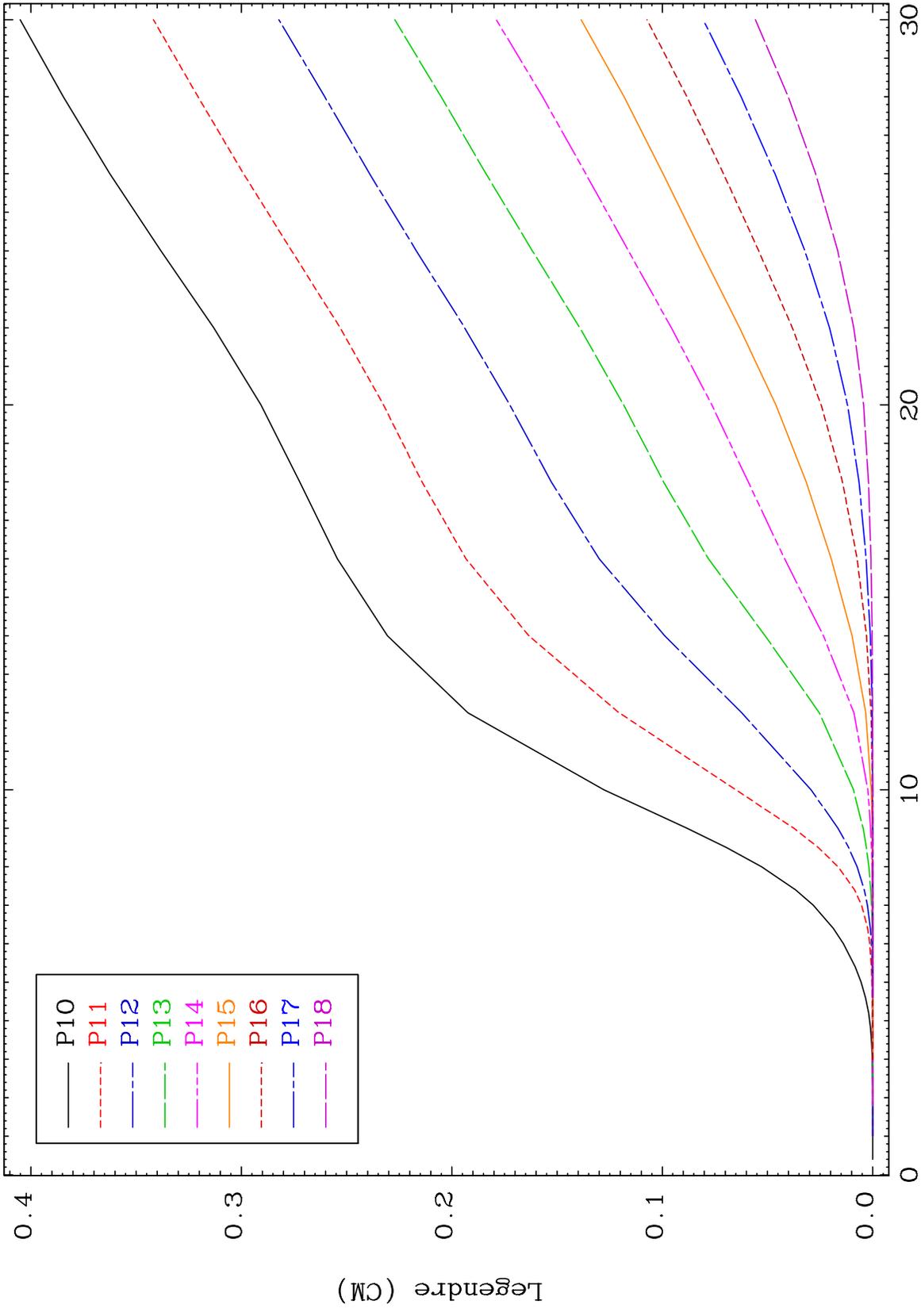
Incident Energy (MeV)

16

MAT 8592

Elastic Legendre Coefficients

86-Rn-200



17

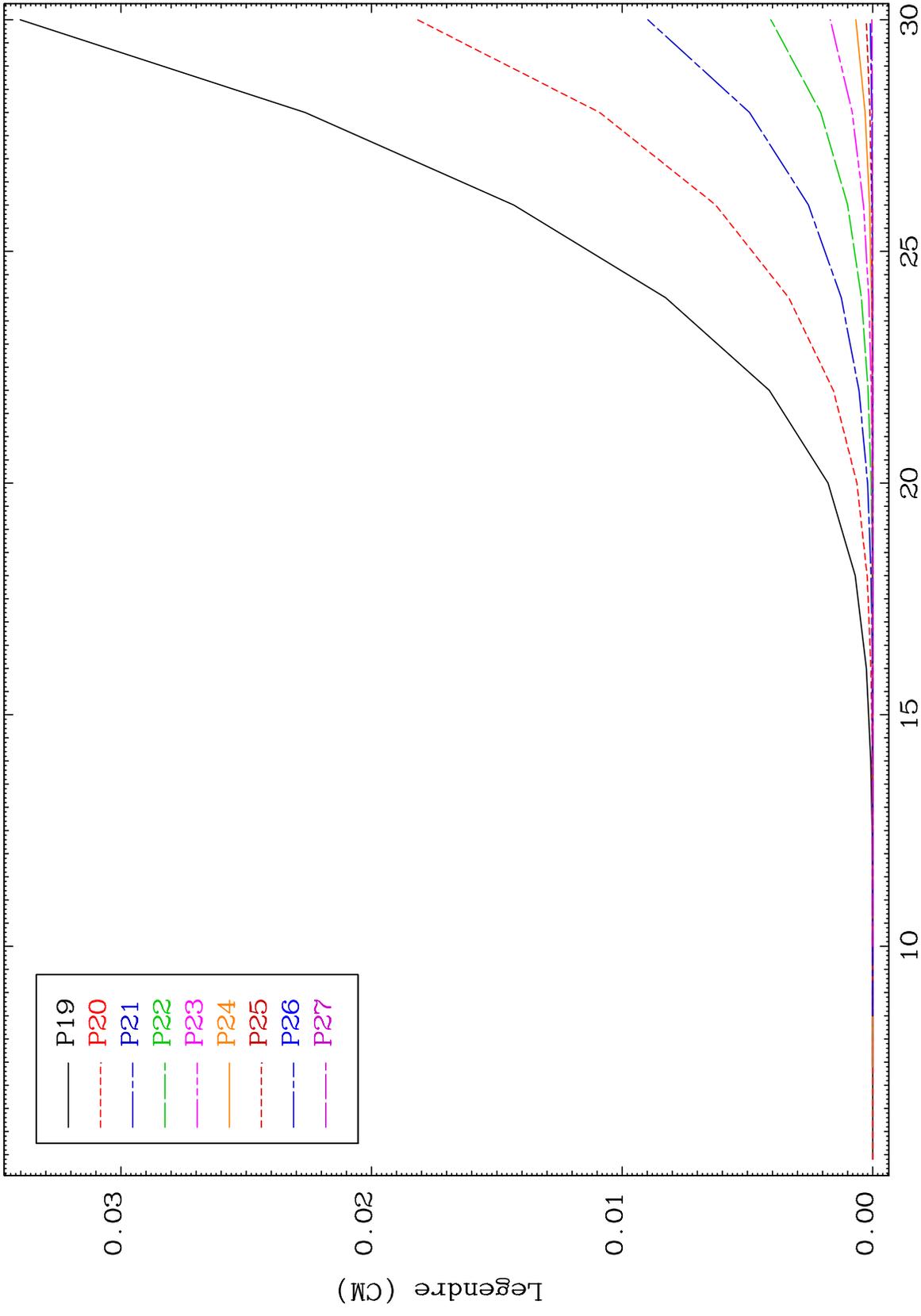
Incident Energy (MeV)

86-Rn-200

MAT 8592

Elastic Legendre Coefficients

86-Rn-200



18

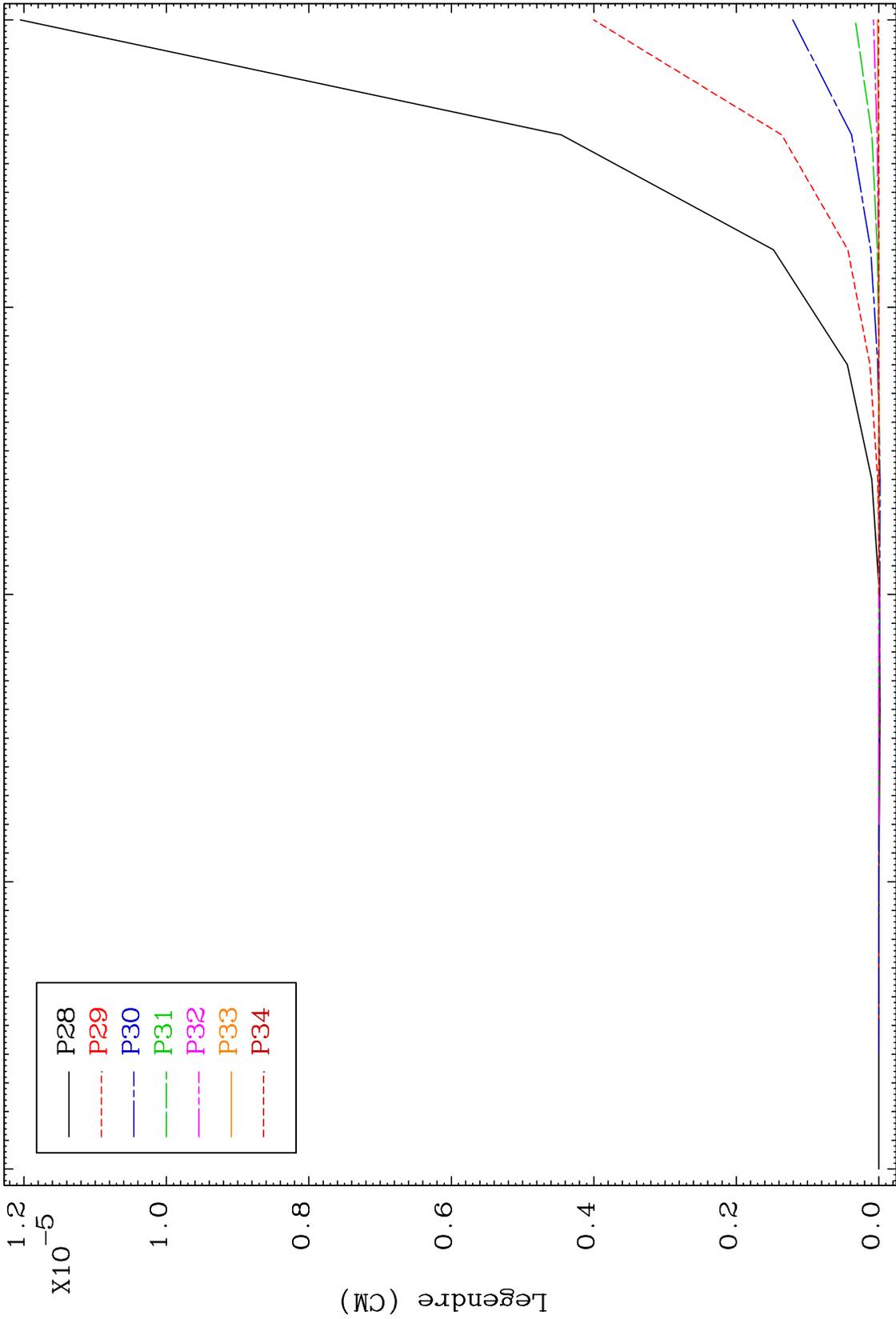
Incident Energy (MeV)

86-Rn-200

MAT 8592

Elastic Legendre Coefficients

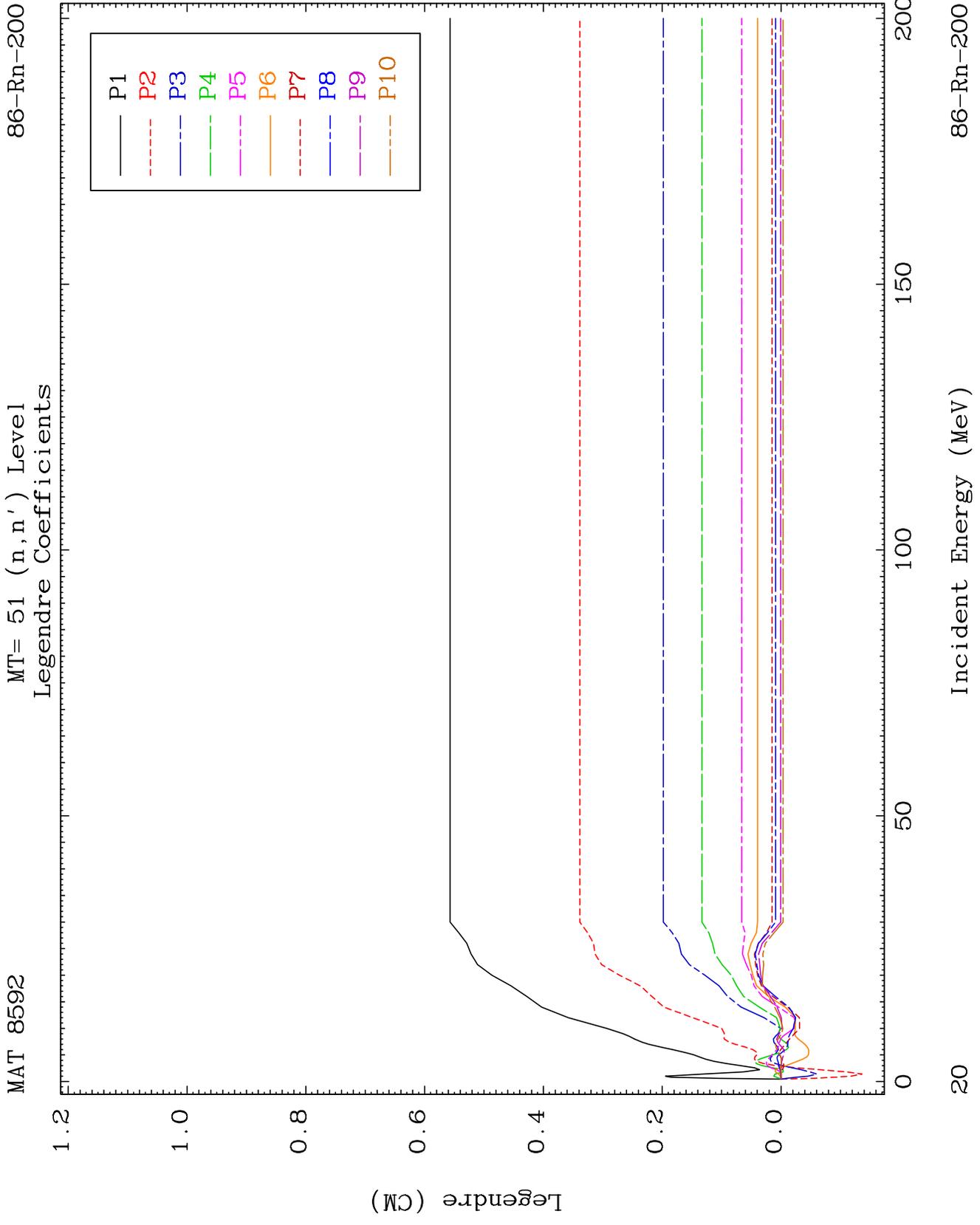
86-Rn-200



19

Incident Energy (MeV)

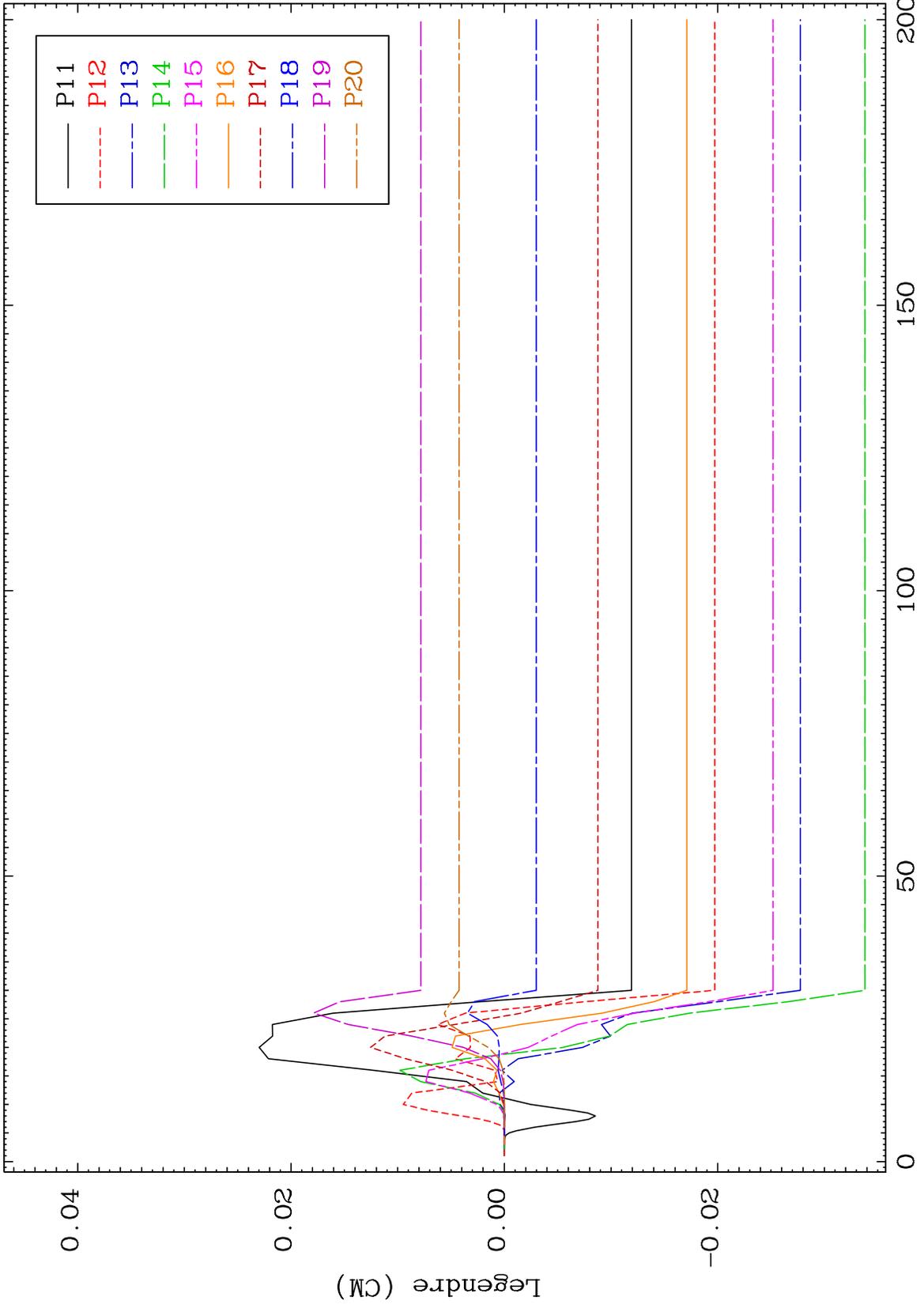
86-Rn-200



MAT 8592

MT= 51 (n,n') Level
Legendre Coefficients

86-Rn-200



21

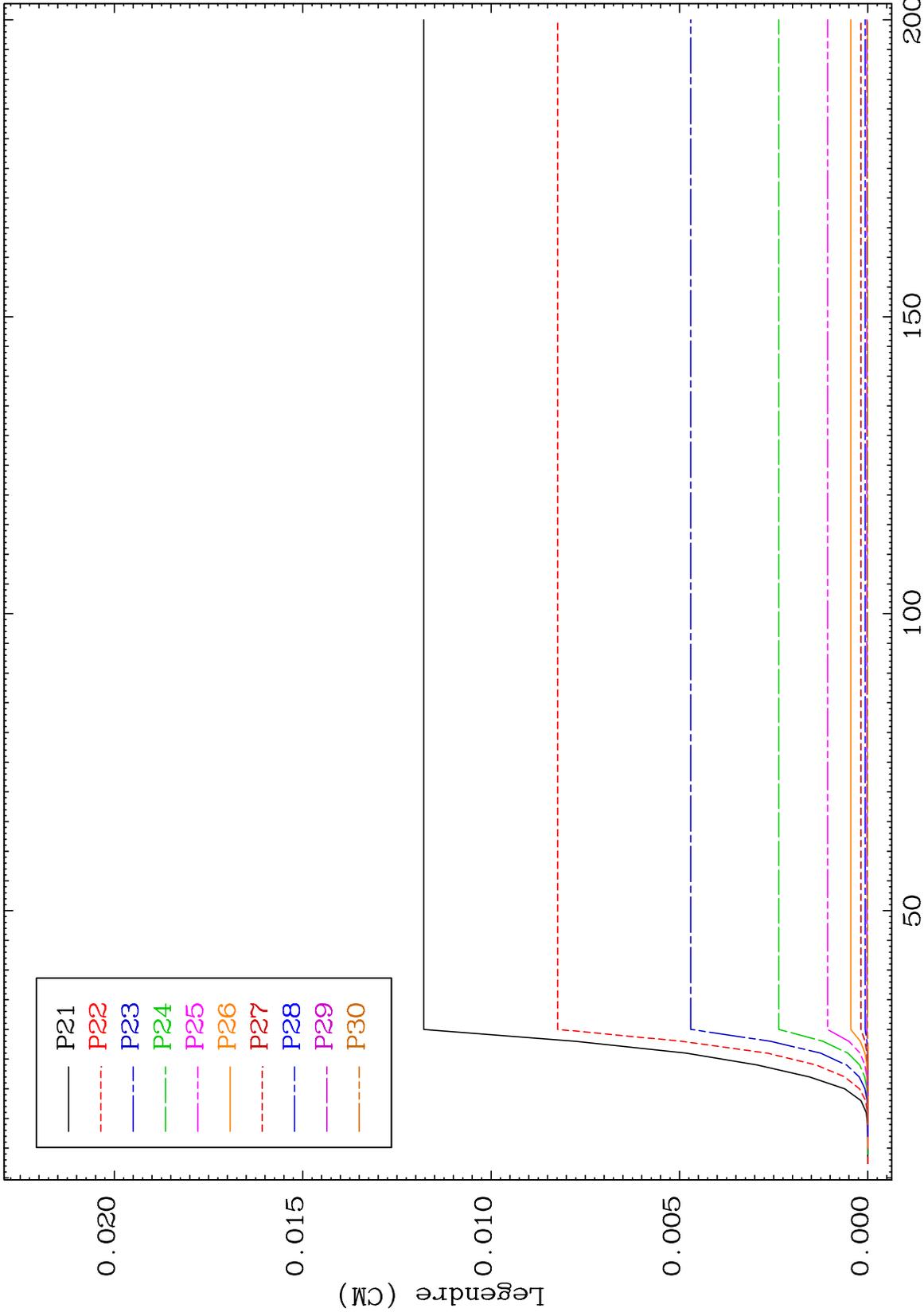
Incident Energy (MeV)

86-Rn-200

MAT 8592

MT= 51 (n,n') Level
Legendre Coefficients

86-Rn-200



22

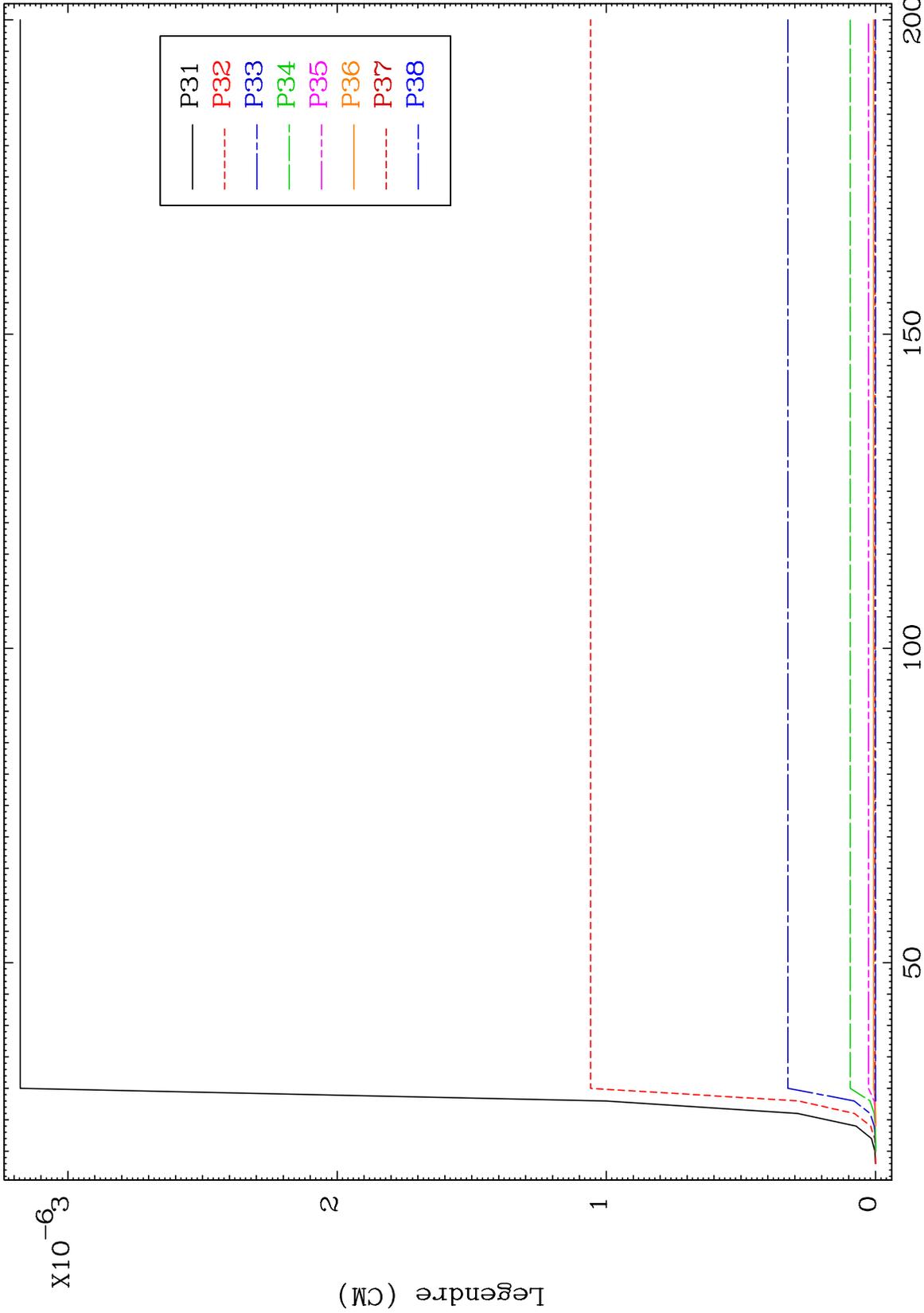
Incident Energy (MeV)

86-Rn-200

MAT 8592

MT= 51 (n,n') Level
Legendre Coefficients

86-Rn-200



23

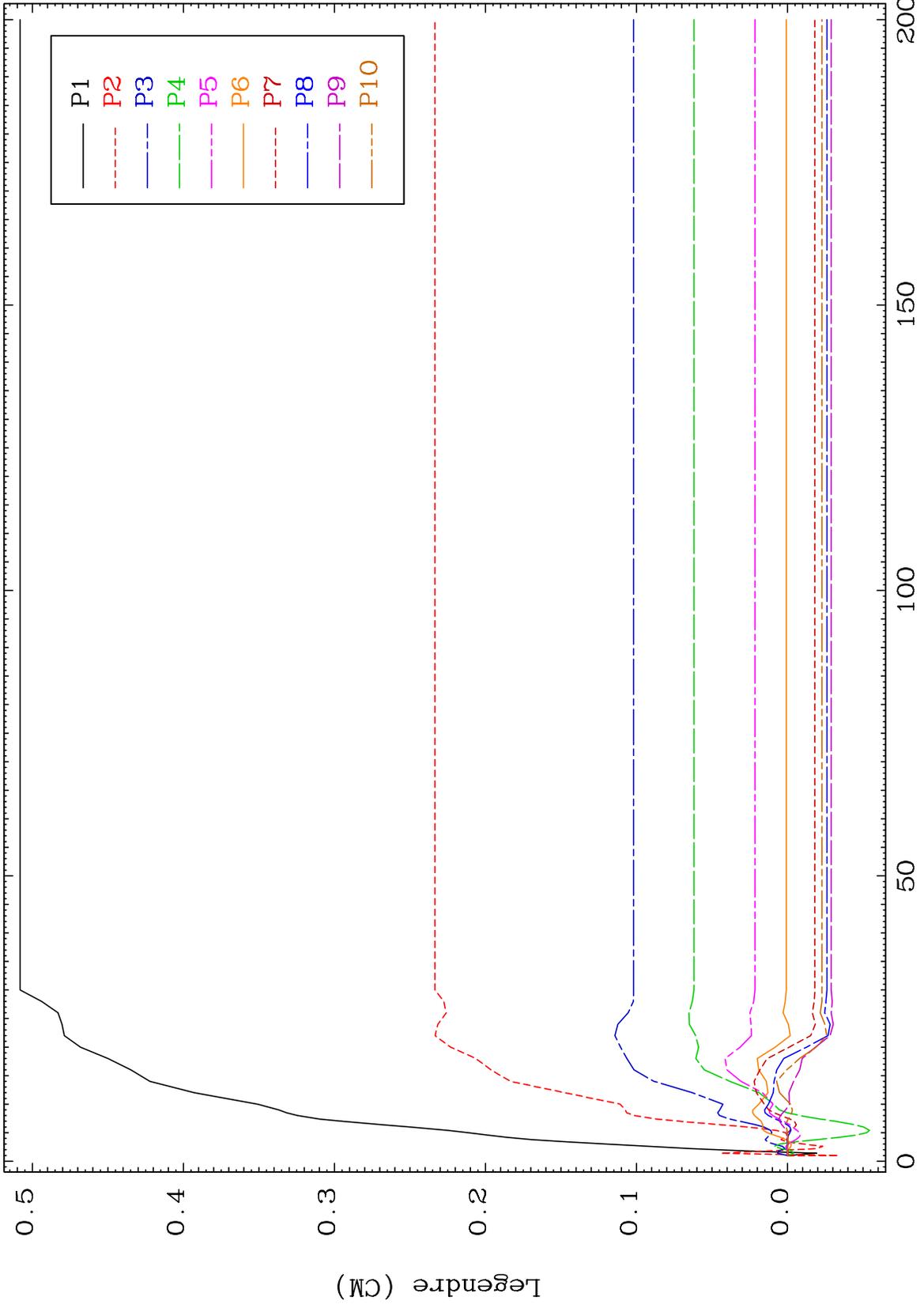
Incident Energy (MeV)

86-Rn-200

MAT 8592

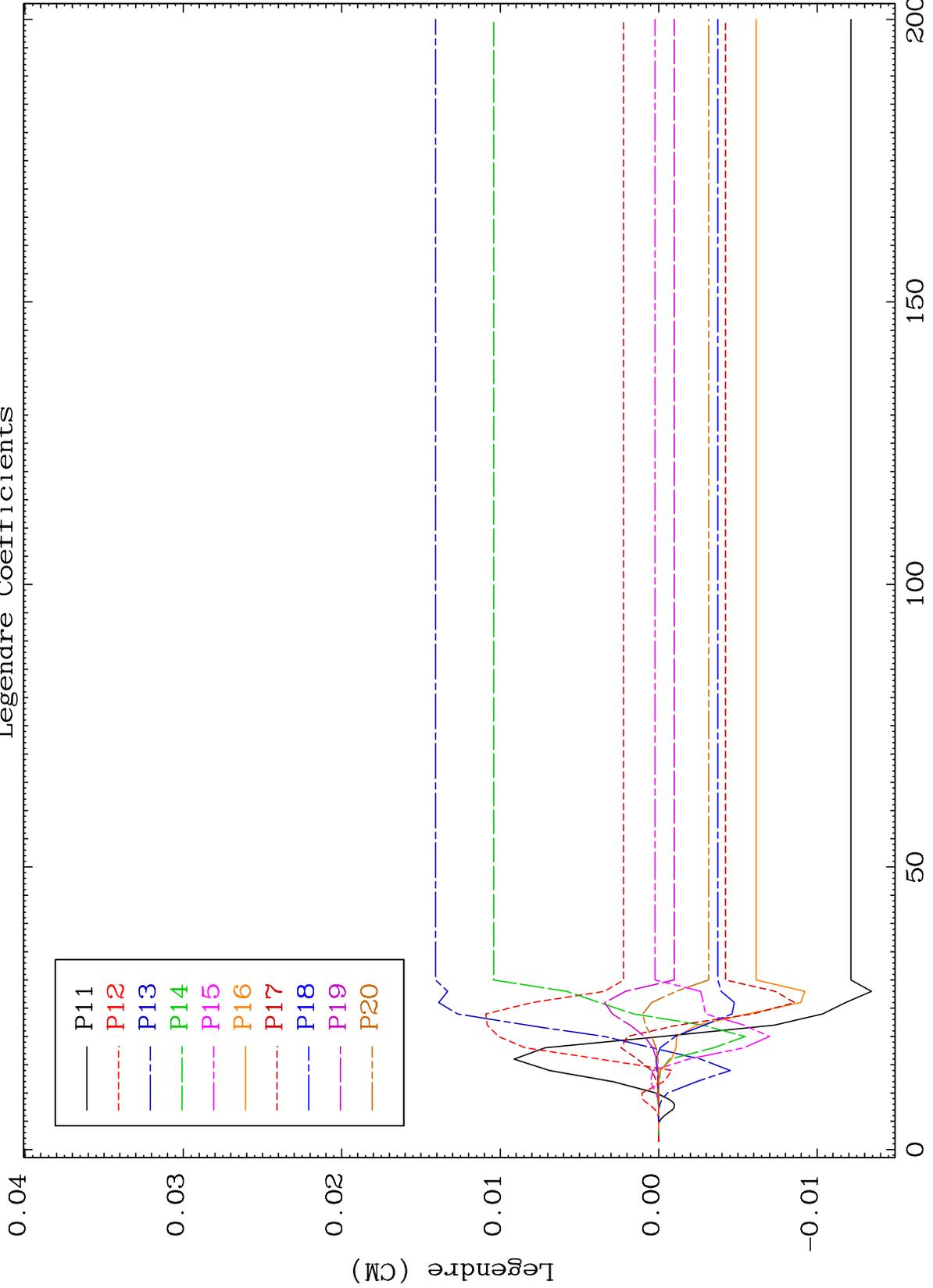
MT= 52 (n,n') Level
Legendre Coefficients

86-Rn-200



24

MAT 8592 MT= 52 (n,n') Level Legendre Coefficients 86-Rn-200



86-Rn-200

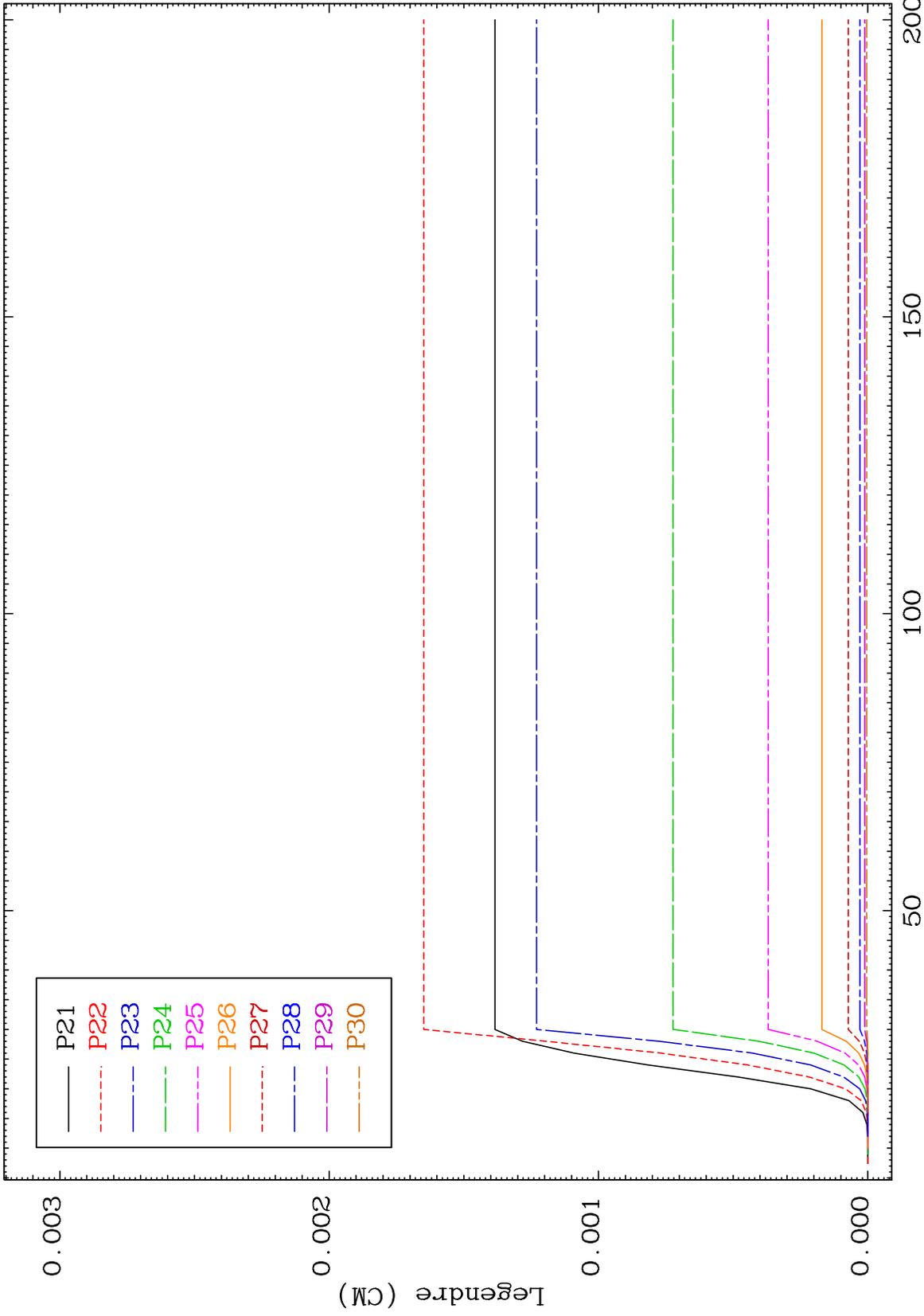
Incident Energy (MeV)

25

MAT 8592

MT= 52 (n,n') Level
Legendre Coefficients

86-Rn-200



26

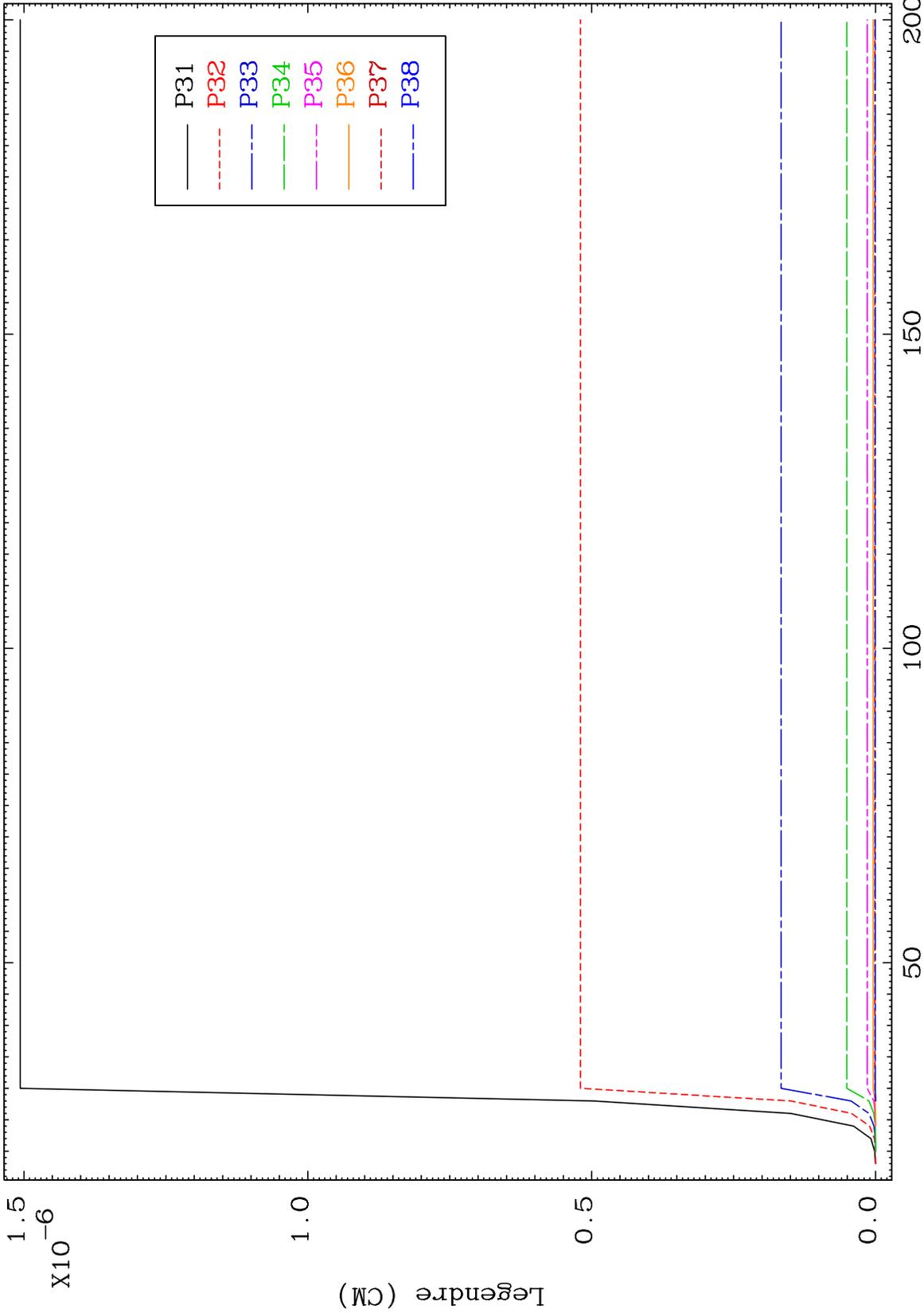
Incident Energy (MeV)

86-Rn-200

MAT 8592

MT= 52 (n,n') Level
Legendre Coefficients

86-Rn-200



27

Incident Energy (MeV)

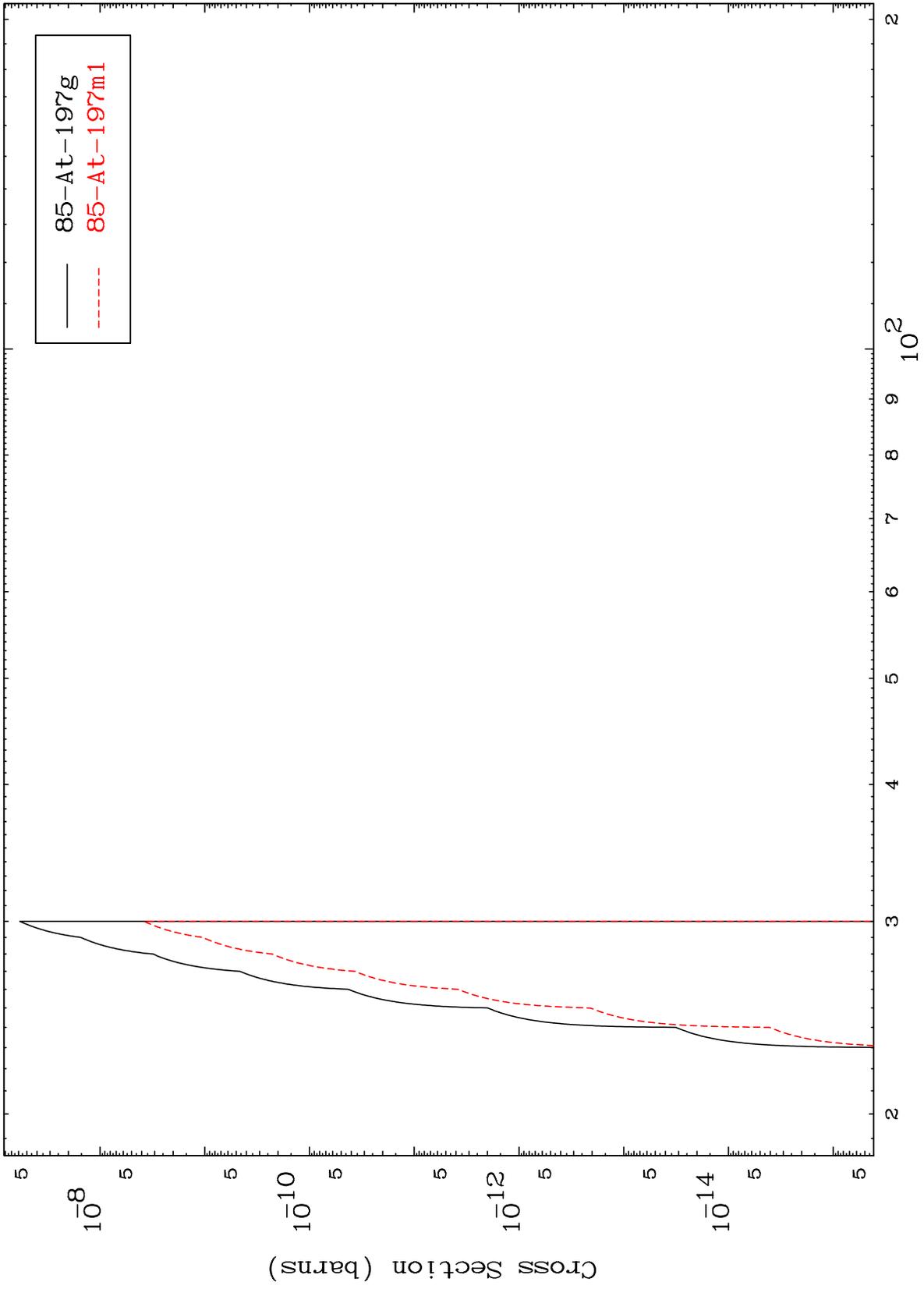
86-Rn-200

MAT 8592

(n,2n) d

86-Rn-200

Radionuclide Production Cross Section



85-At-197g
85-At-197m1

28

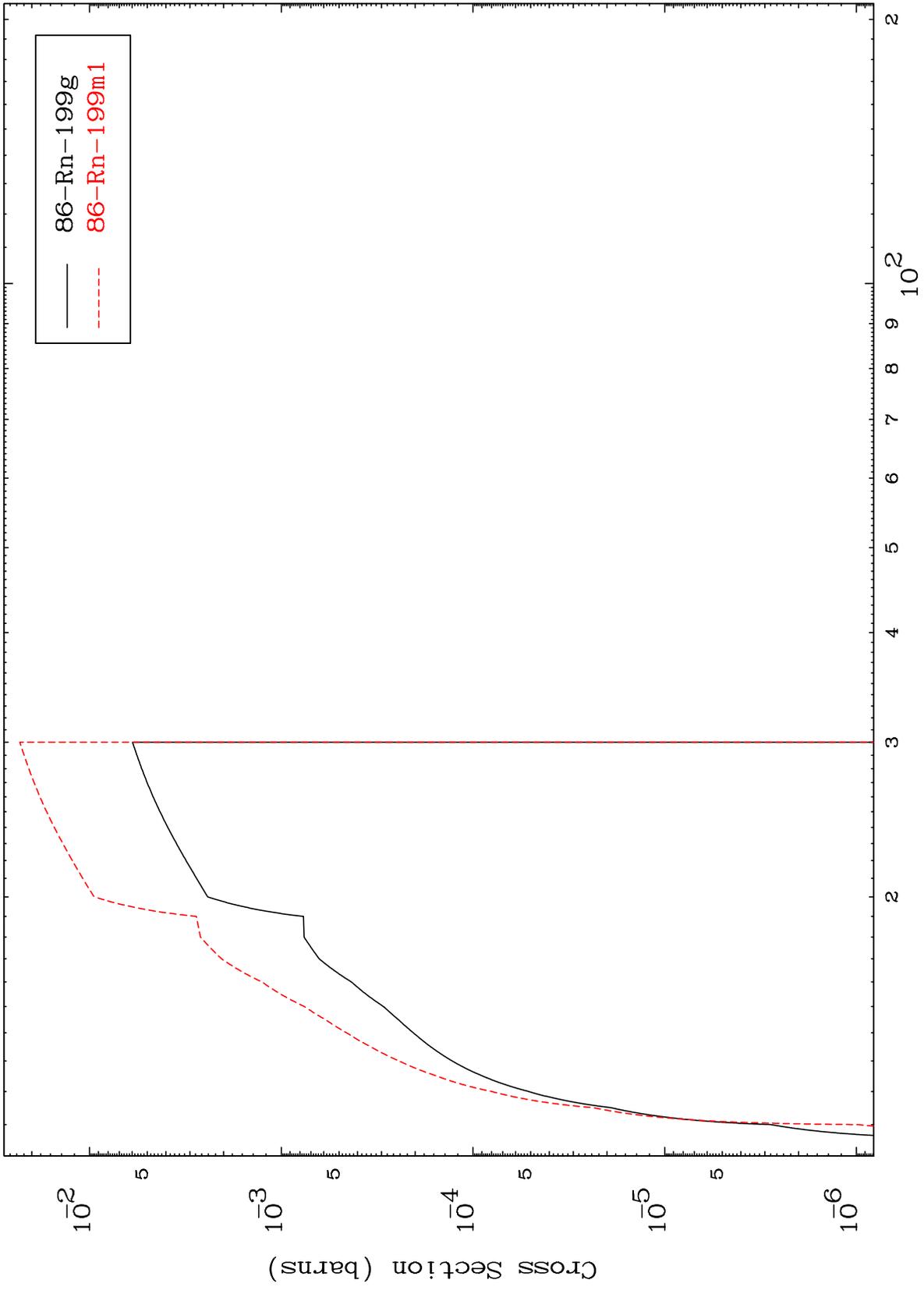
Incident Energy (MeV)

86-Rn-200

MAT 8592

86-Rn-200

(n,2n)
Radionuclide Production Cross Section



29

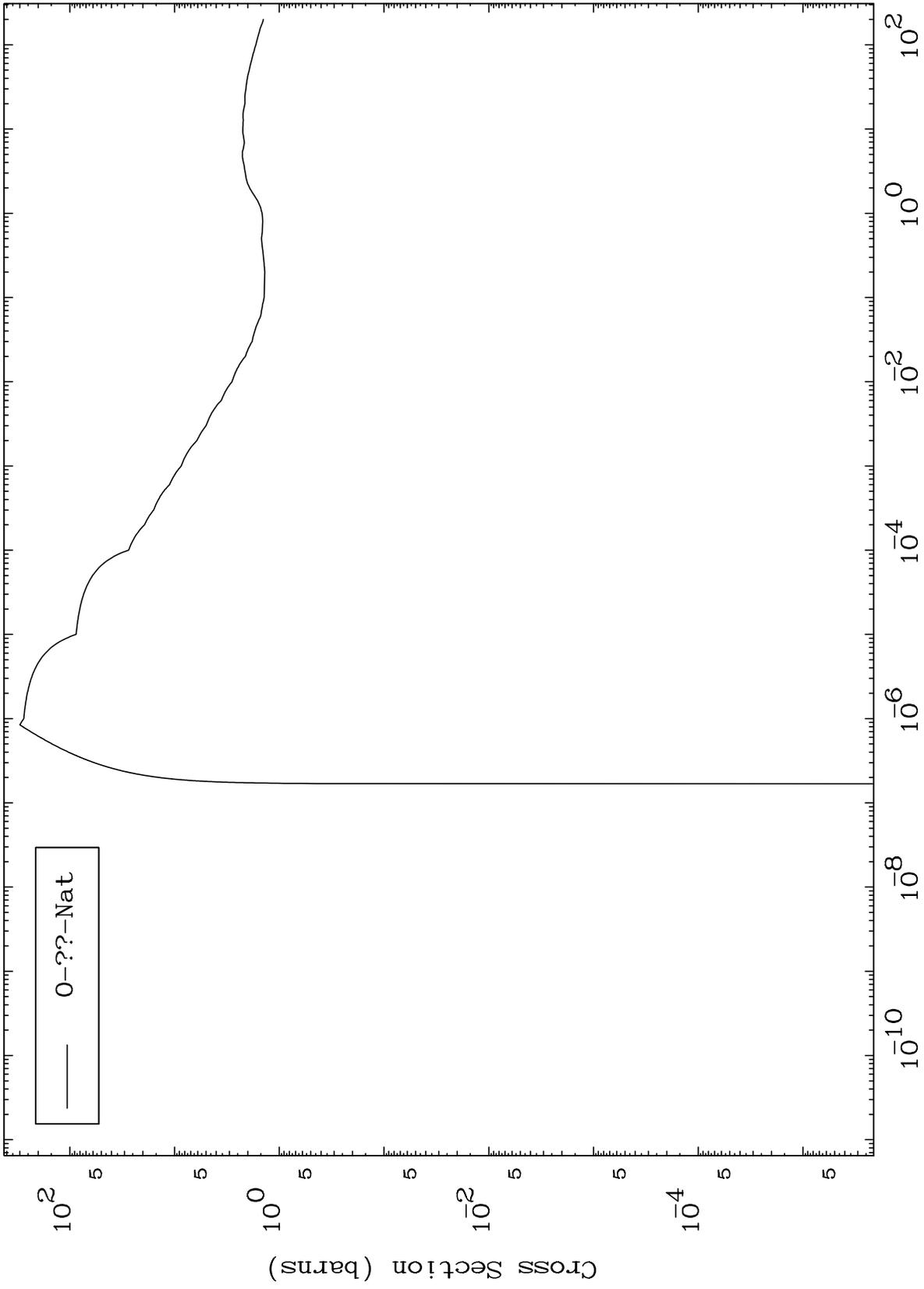
Incident Energy (MeV)

86-Rn-200

MAT 8592

86-Rn-200

Fission
Radionuclide Production Cross Section



30

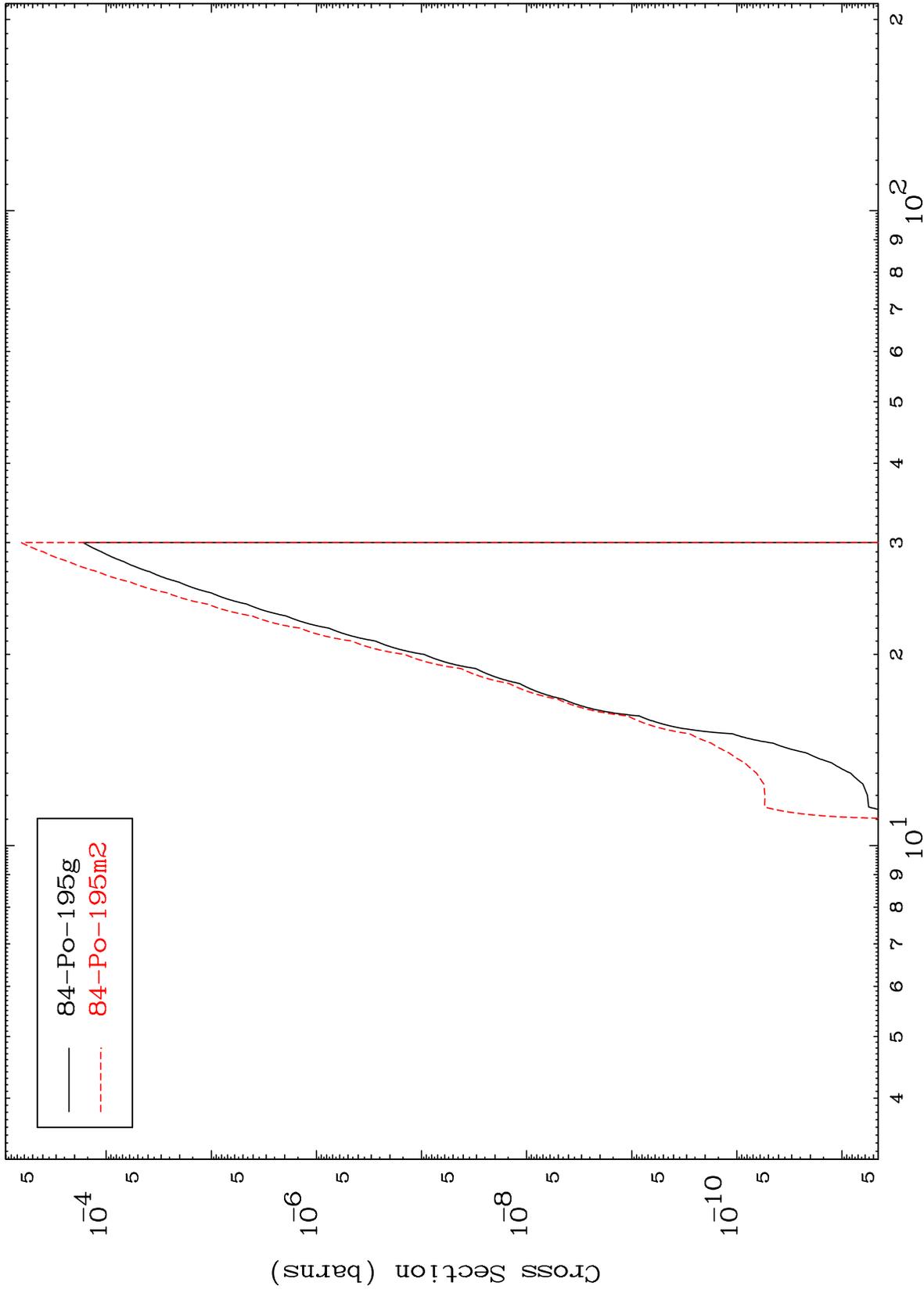
86-Rn-200

MAT 8592

(n,2n) α

86-Rn-200

Radionuclide Production Cross Section



31

Incident Energy (MeV)

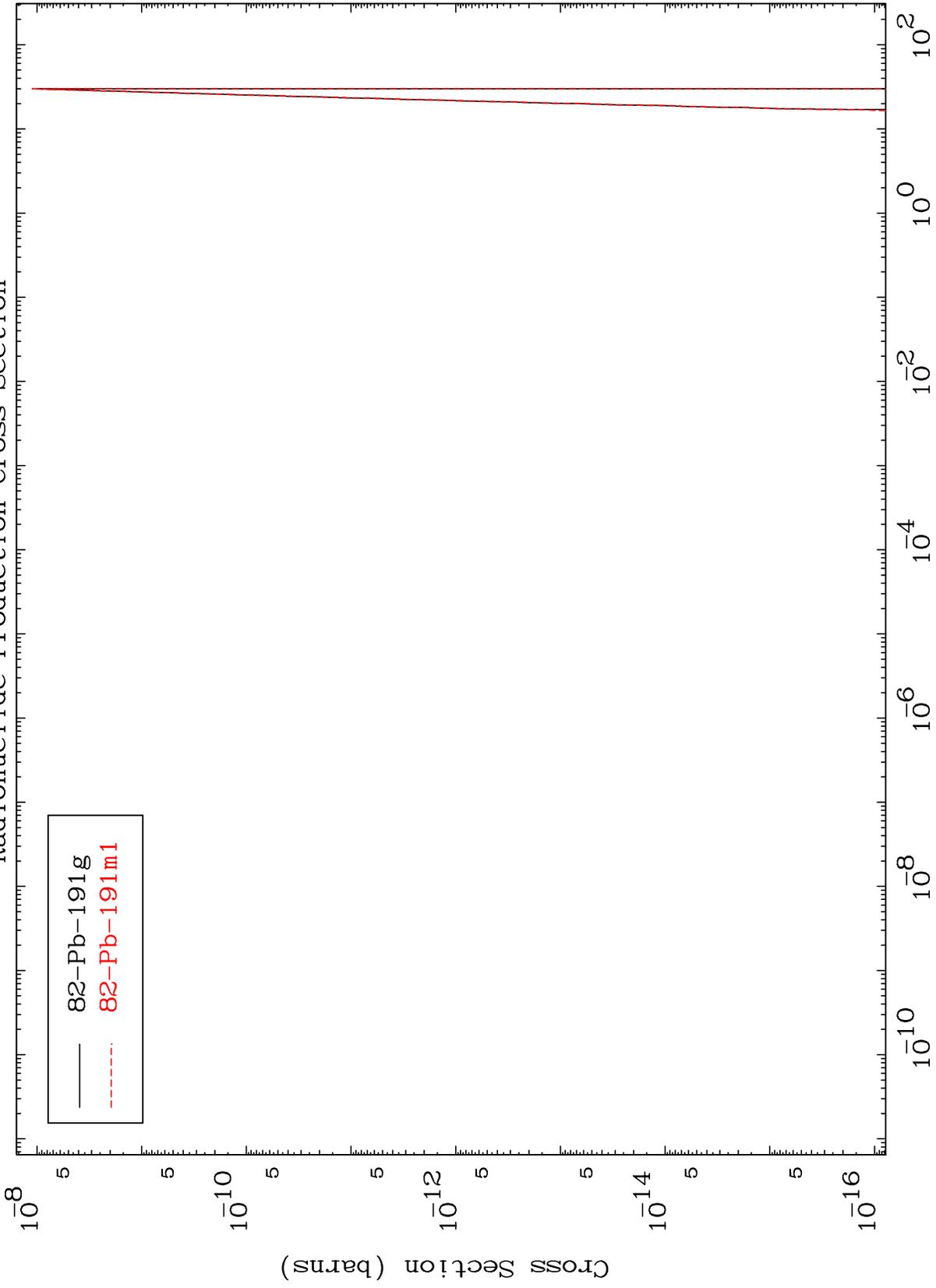
86-Rn-200

MAT 8592

(n,2n) 2 α

86-Rn-200

Radionuclide Production Cross Section



32

Incident Energy (MeV)

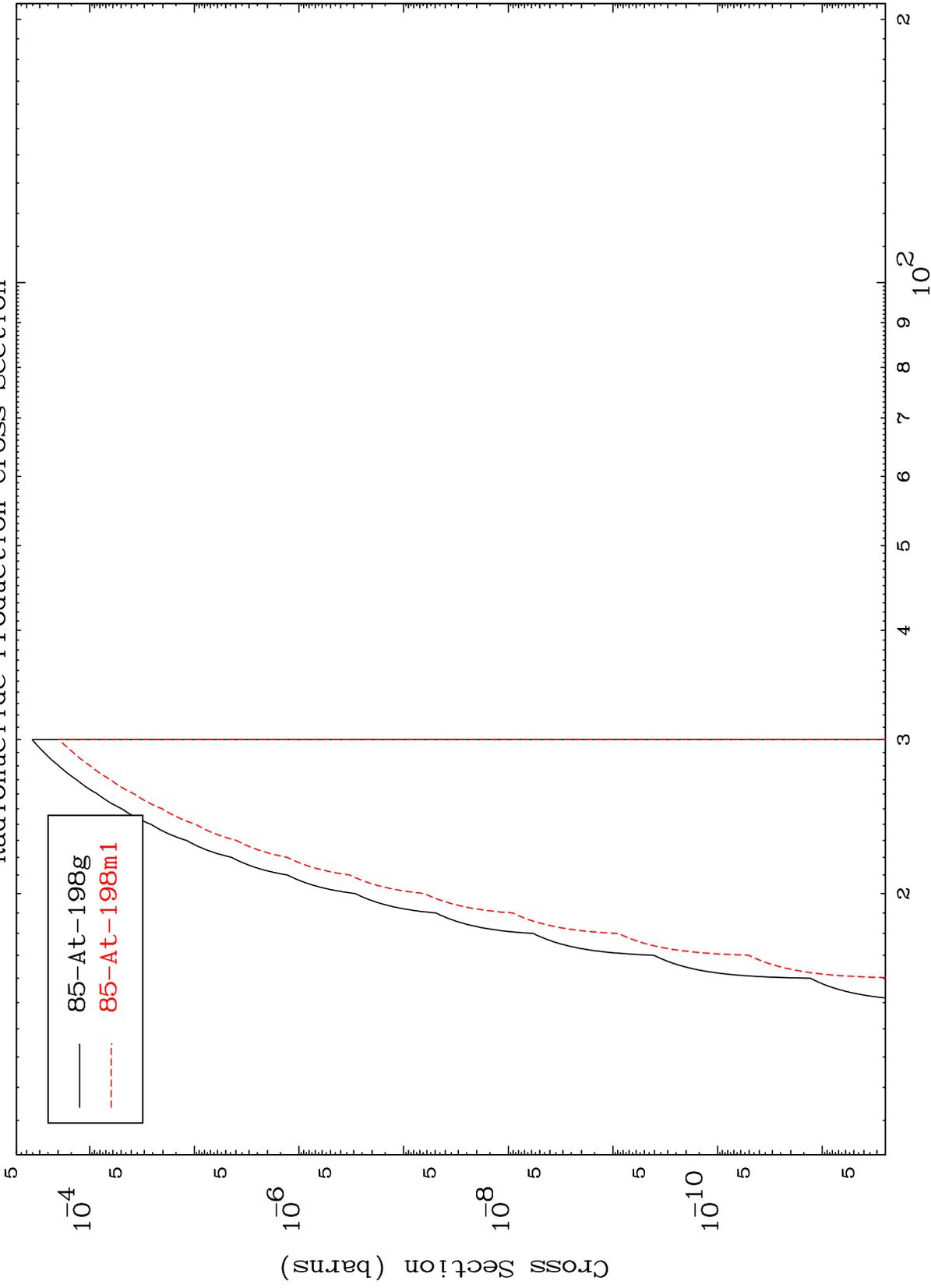
86-Rn-200

MAT 8592

(n,n') d

86-Rn-200

Radionuclide Production Cross Section



33

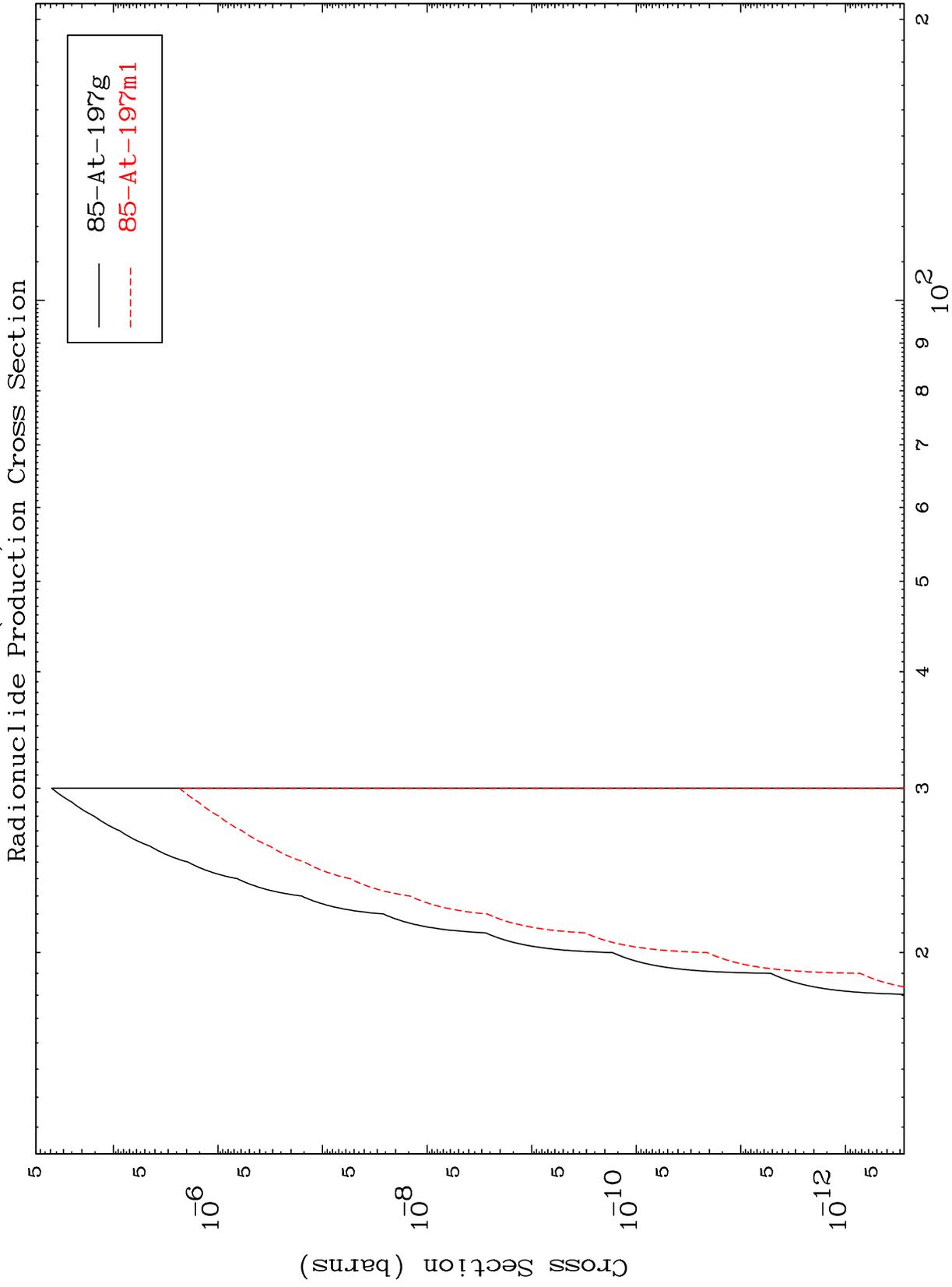
Incident Energy (MeV)

86-Rn-200

MAT 8592

(n,n') t

86-Rn-200



34

86-Rn-200

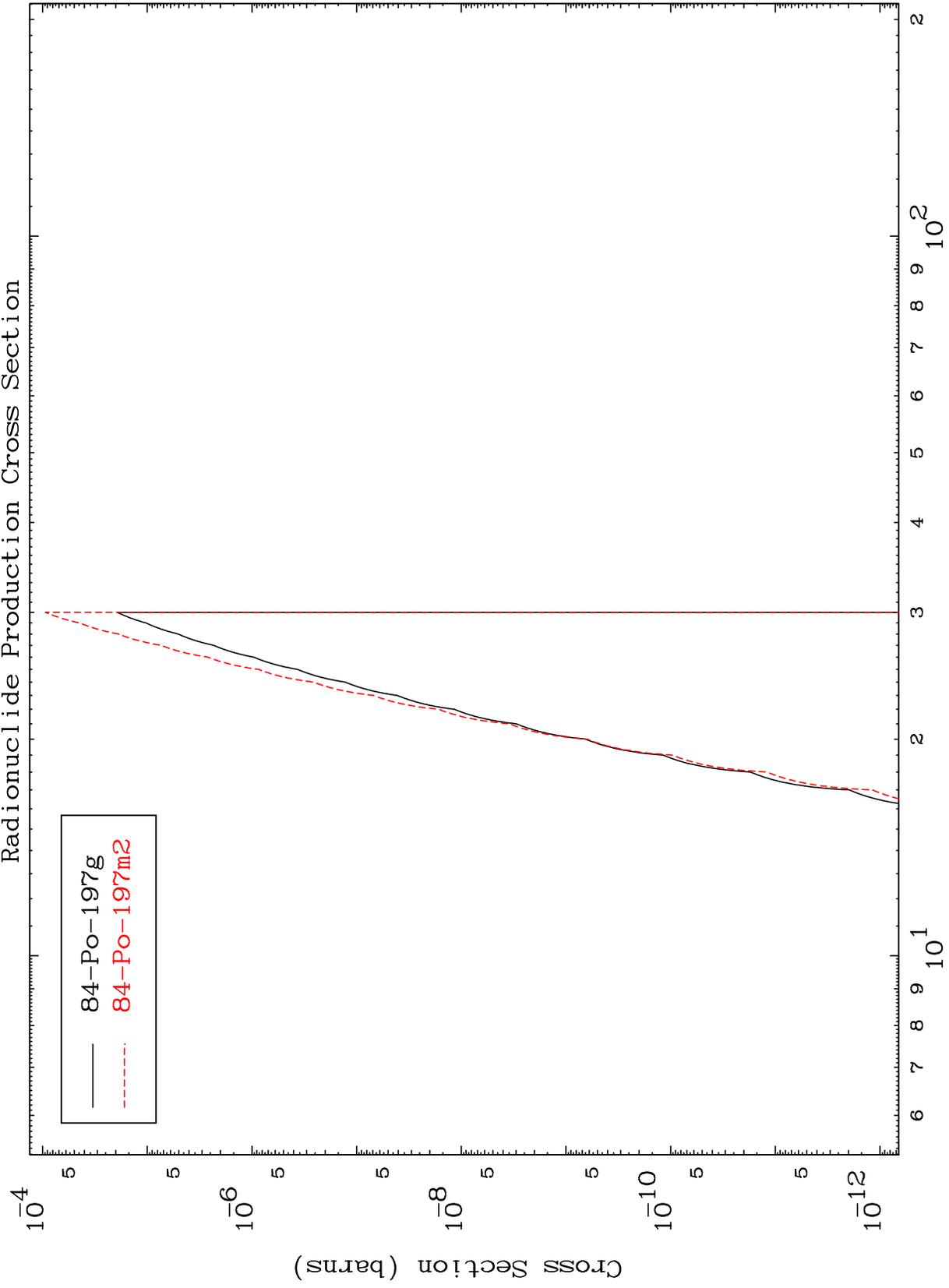
86-Rn-200

MAT 8592

(n, n') He-3

86-Rn-200

Radionuclide Production Cross Section



35

Incident Energy (MeV)

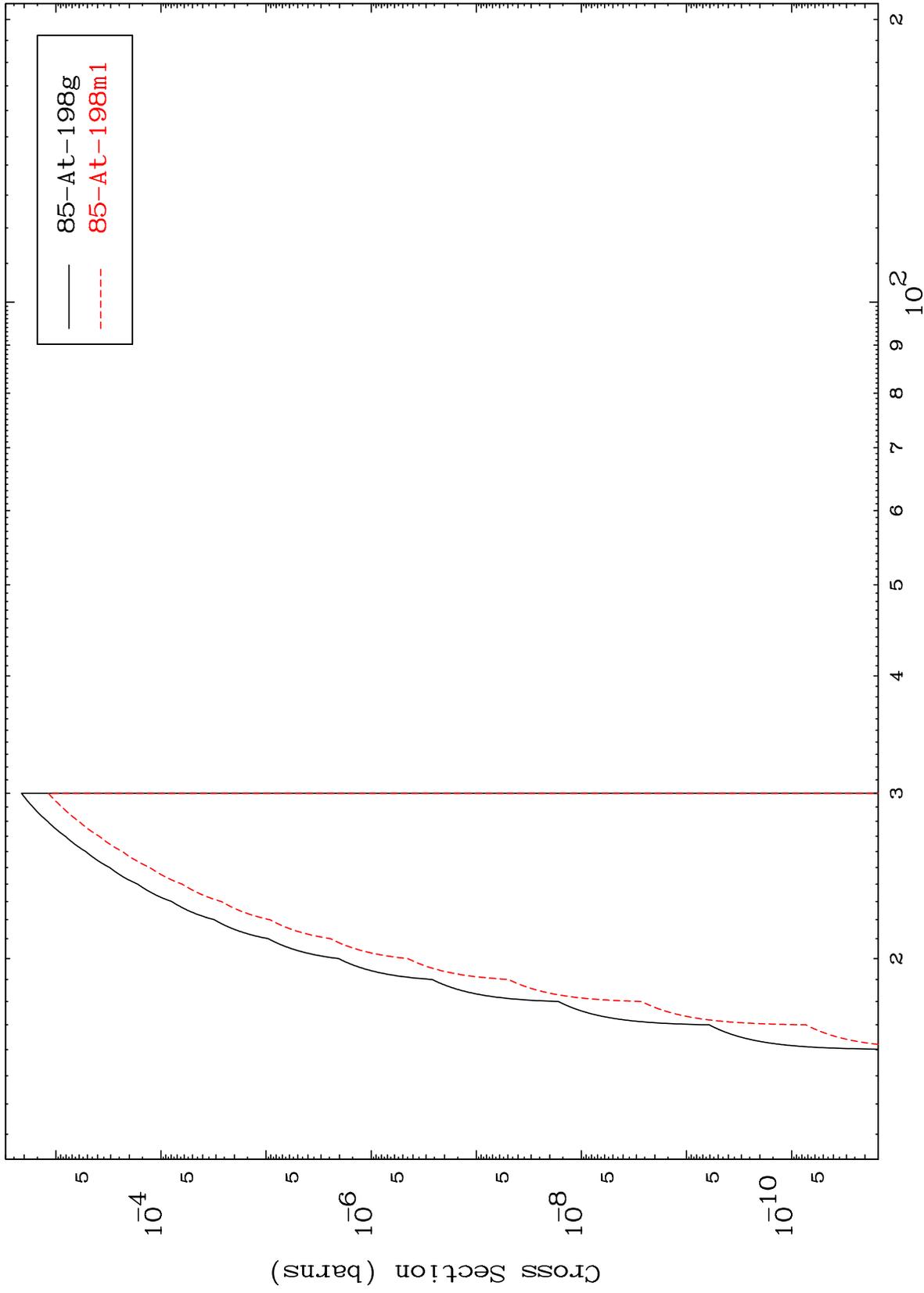
86-Rn-200

MAT 8592

(n,2n) p

86-Rn-200

Radionuclide Production Cross Section



36

Incident Energy (MeV)

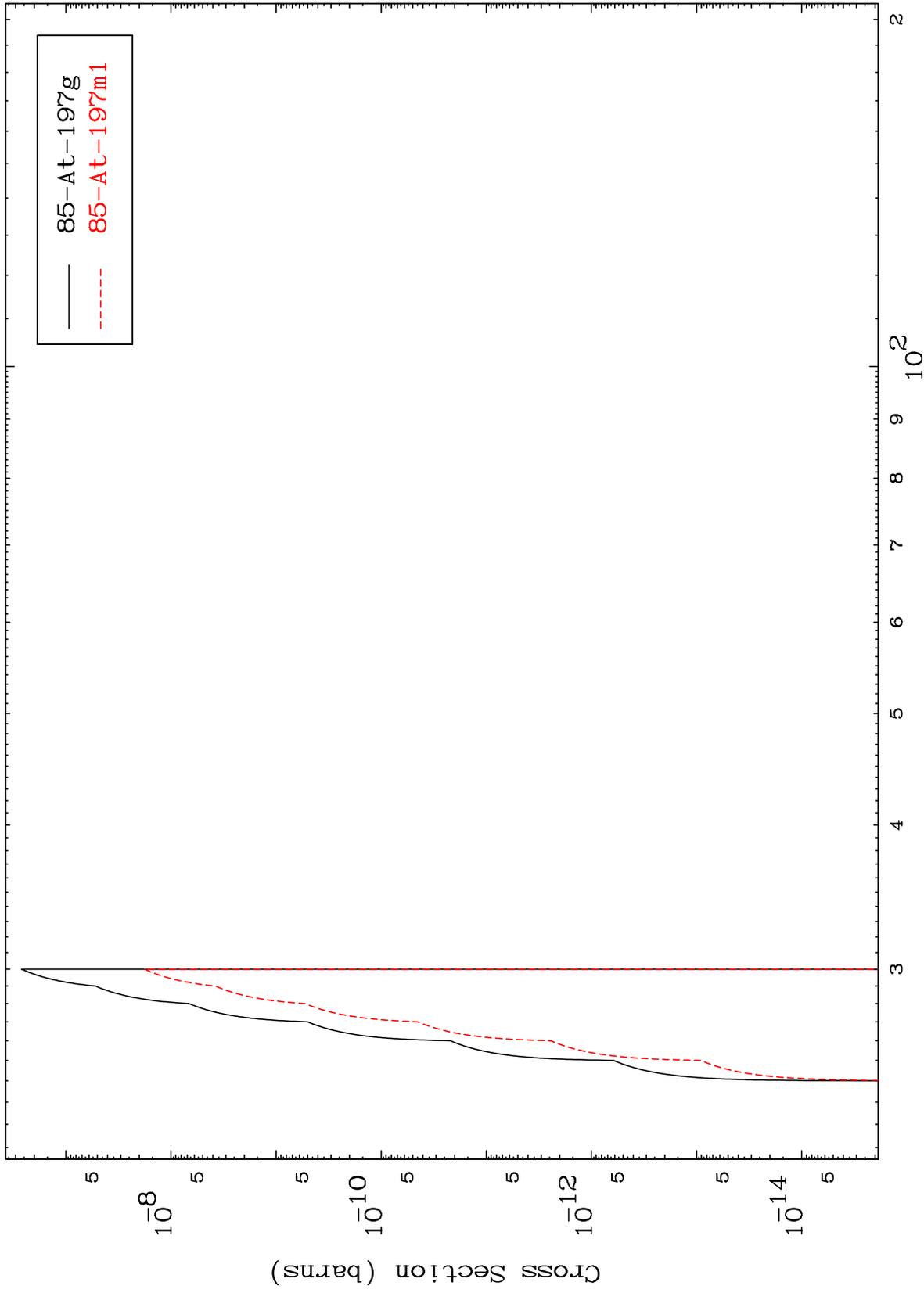
86-Rn-200

MAT 8592

(n,3n) p

86-Rn-200

Radionuclide Production Cross Section



37

Incident Energy (MeV)

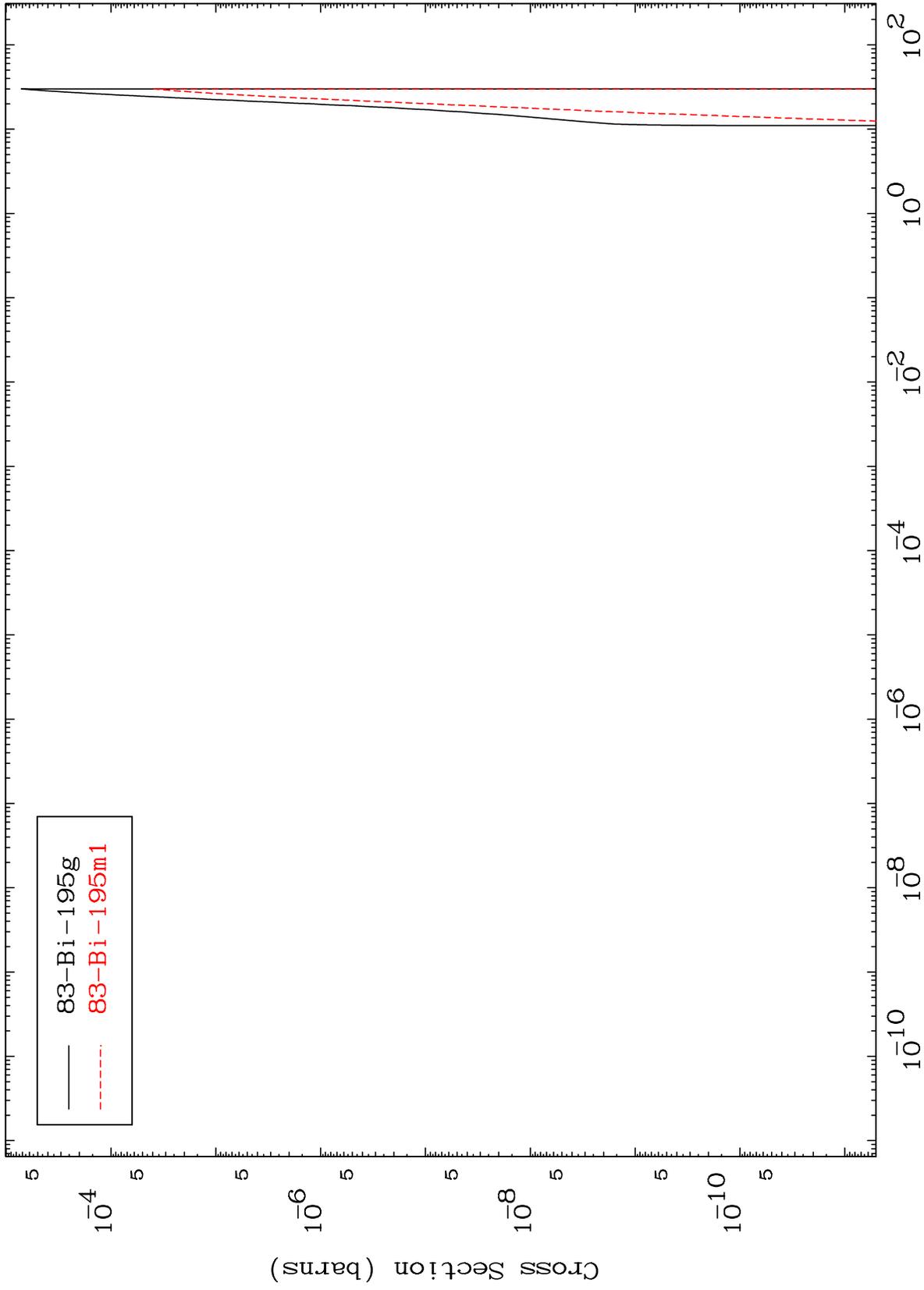
86-Rn-200

MAT 8592

(n,n') p α

86-Rn-200

Radionuclide Production Cross Section



38

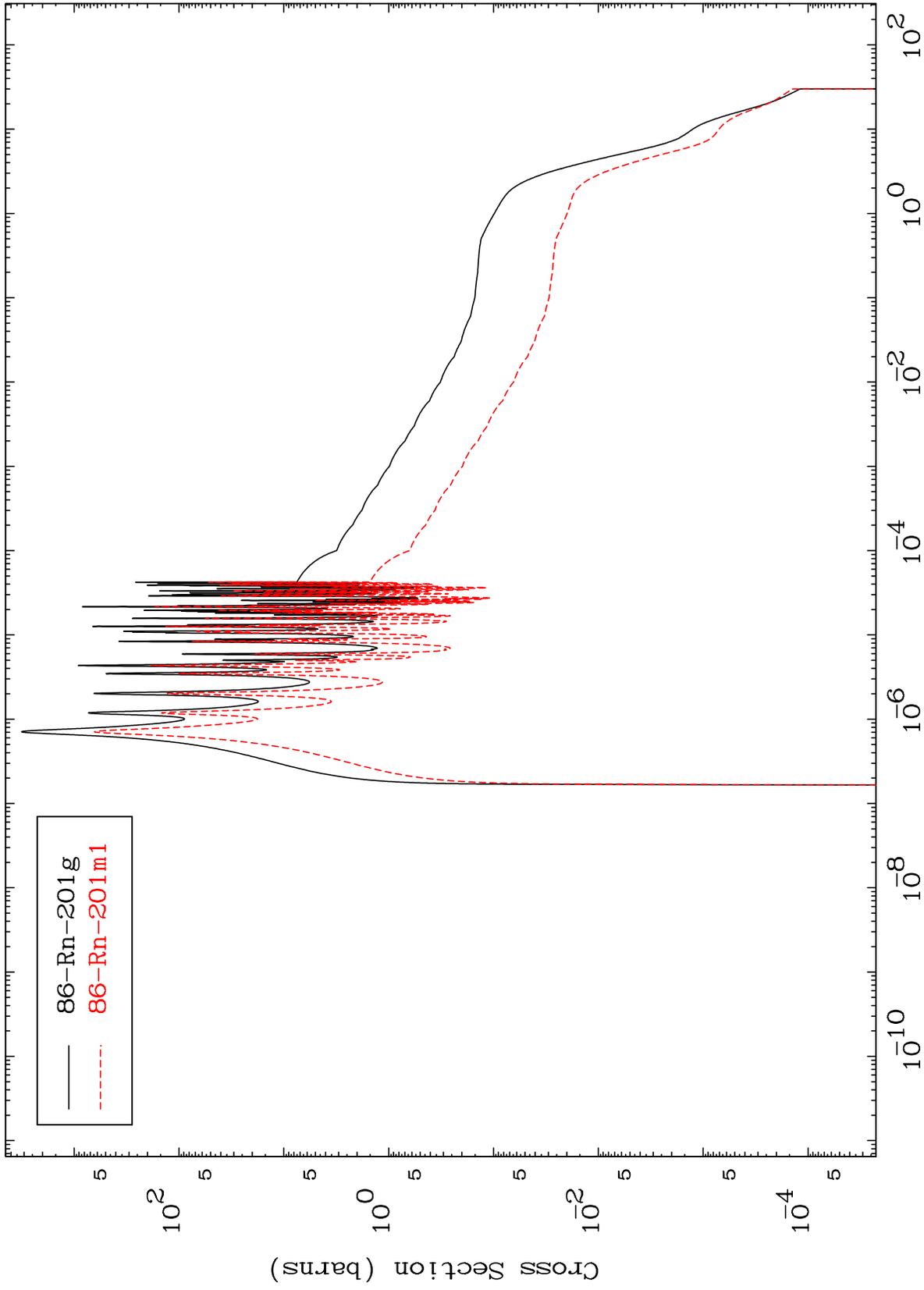
Incident Energy (MeV)

86-Rn-200

MAT 8592

86-Rn-200

Radionuclide Production Cross Section



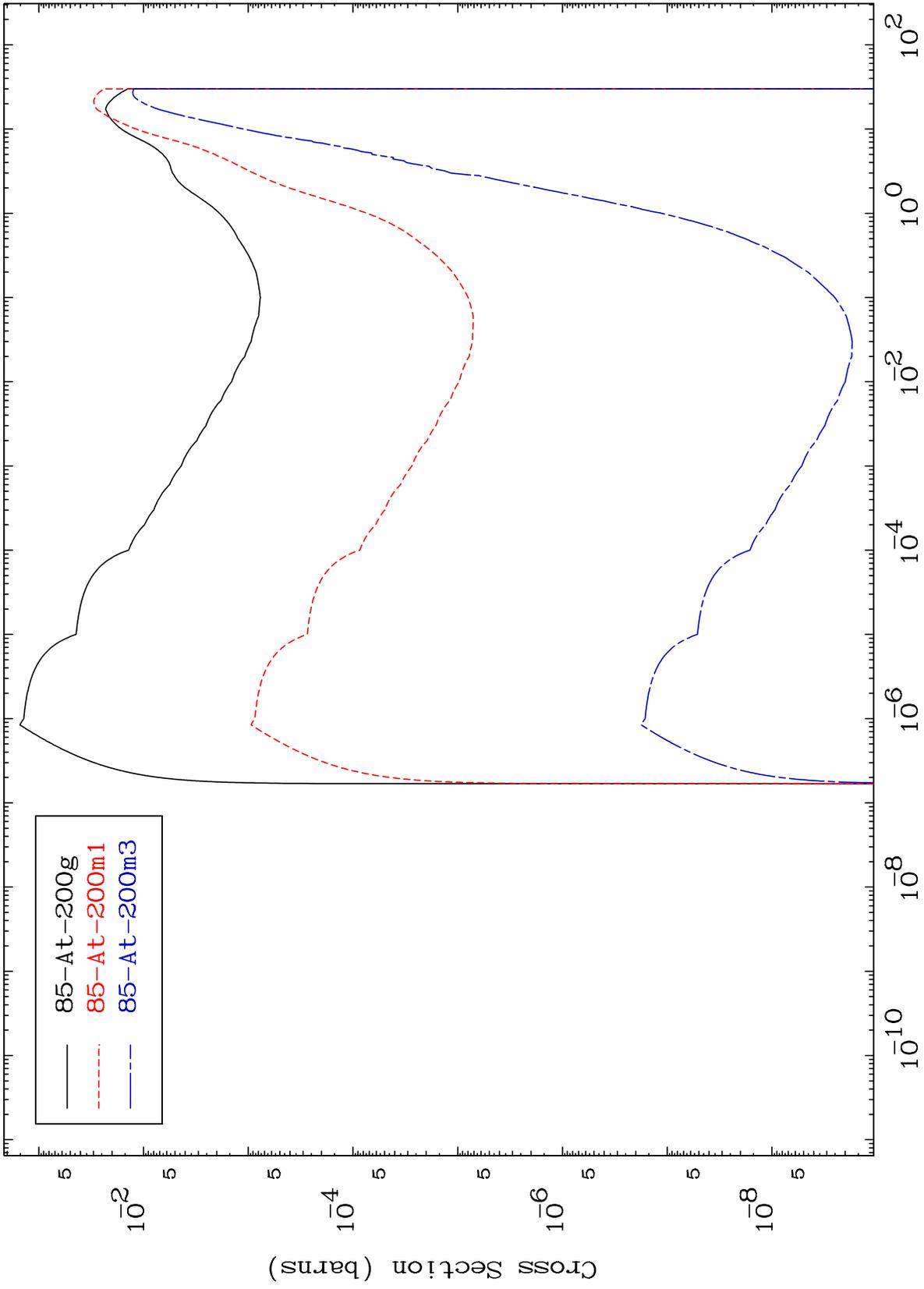
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86-Rn-200

MAT 8592

86-Rn-200

(n,p)
Radionuclide Production Cross Section



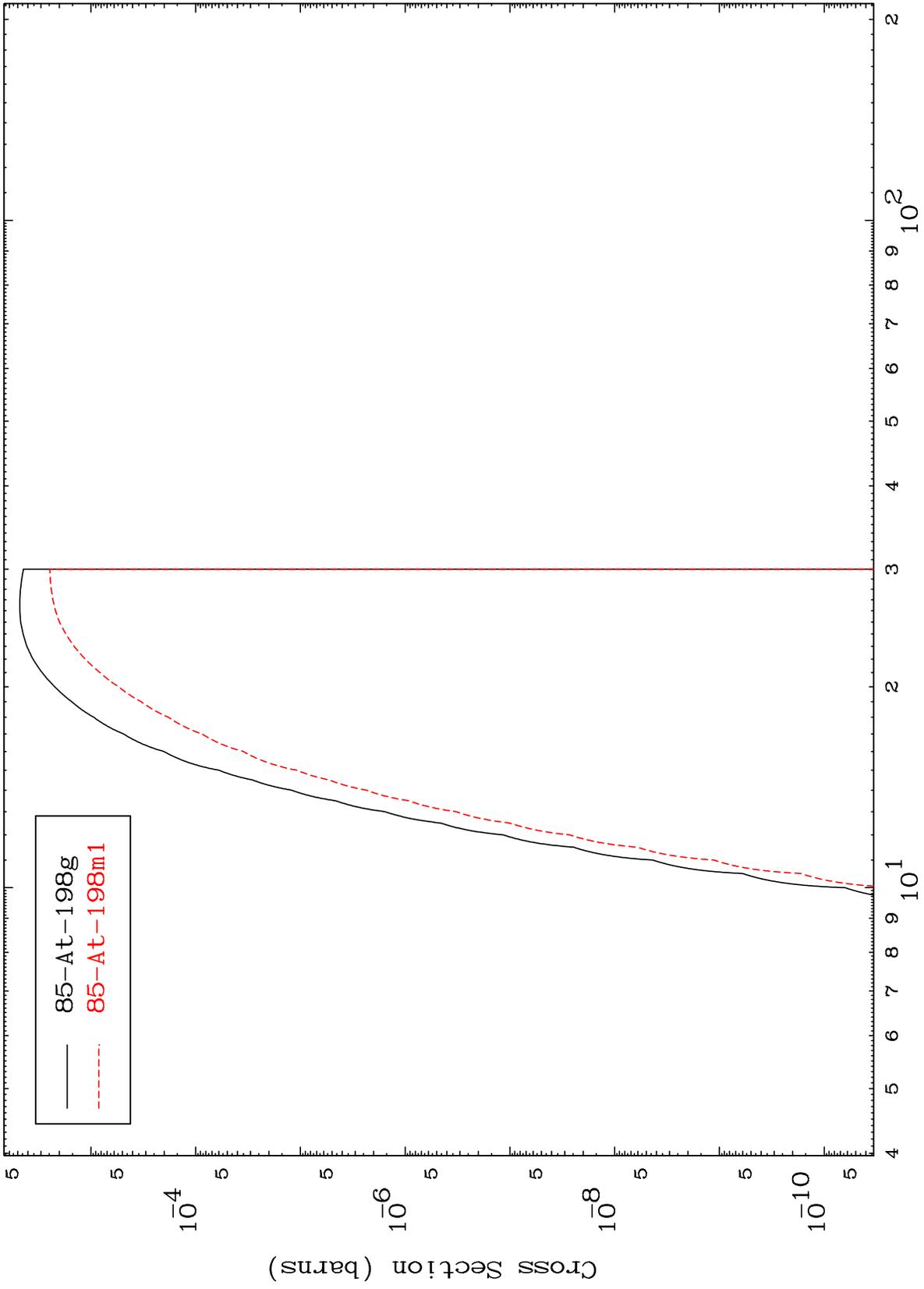
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86-Rn-200

MAT 8592

86-Rn-200

(n, t)
Radionuclide Production Cross Section



41

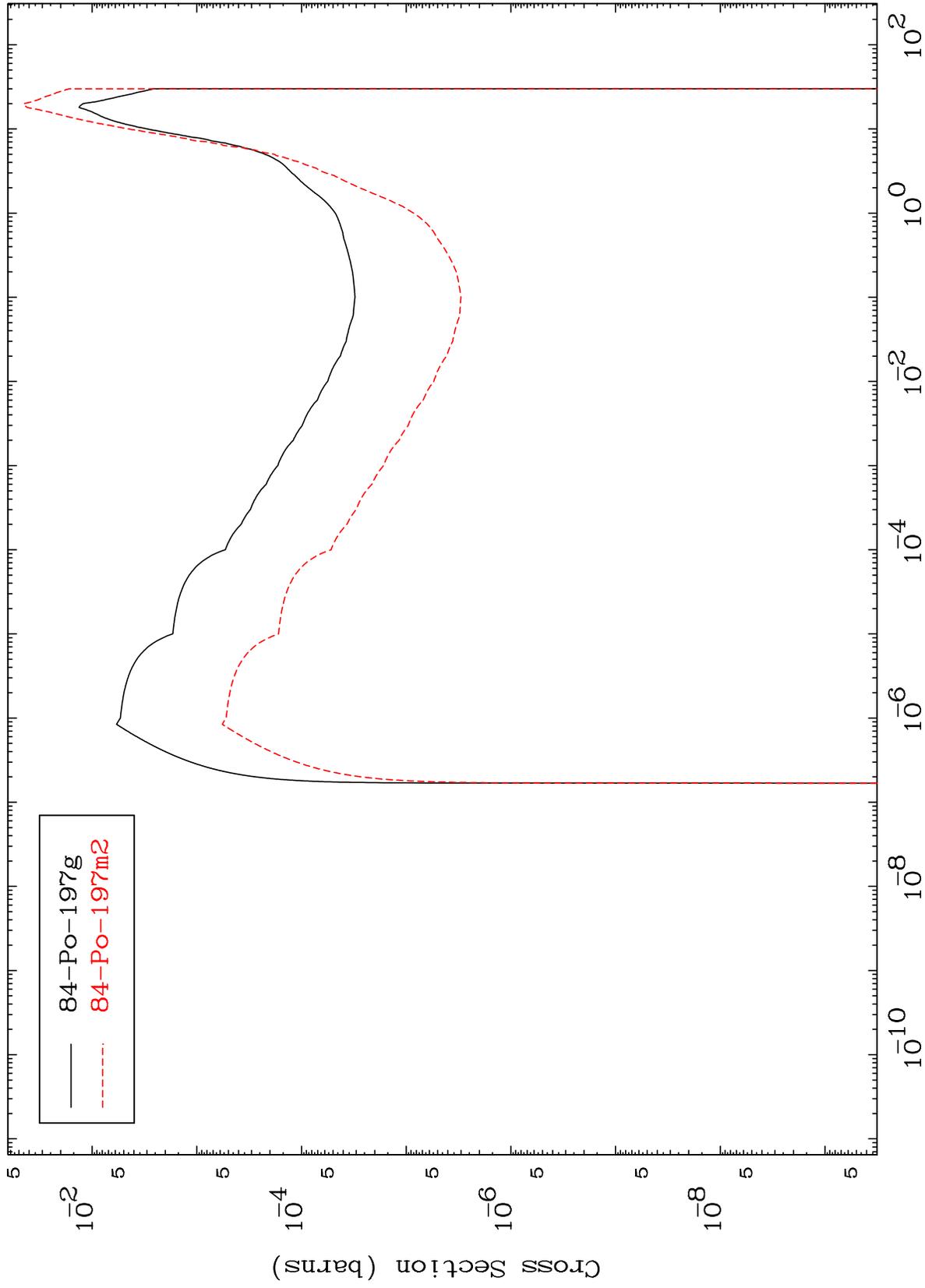
Incident Energy (MeV)

86-Rn-200

MAT 8592

Radionuclide Production Cross Section
(n, α)

86-Rn-200



42

Incident Energy (MeV)

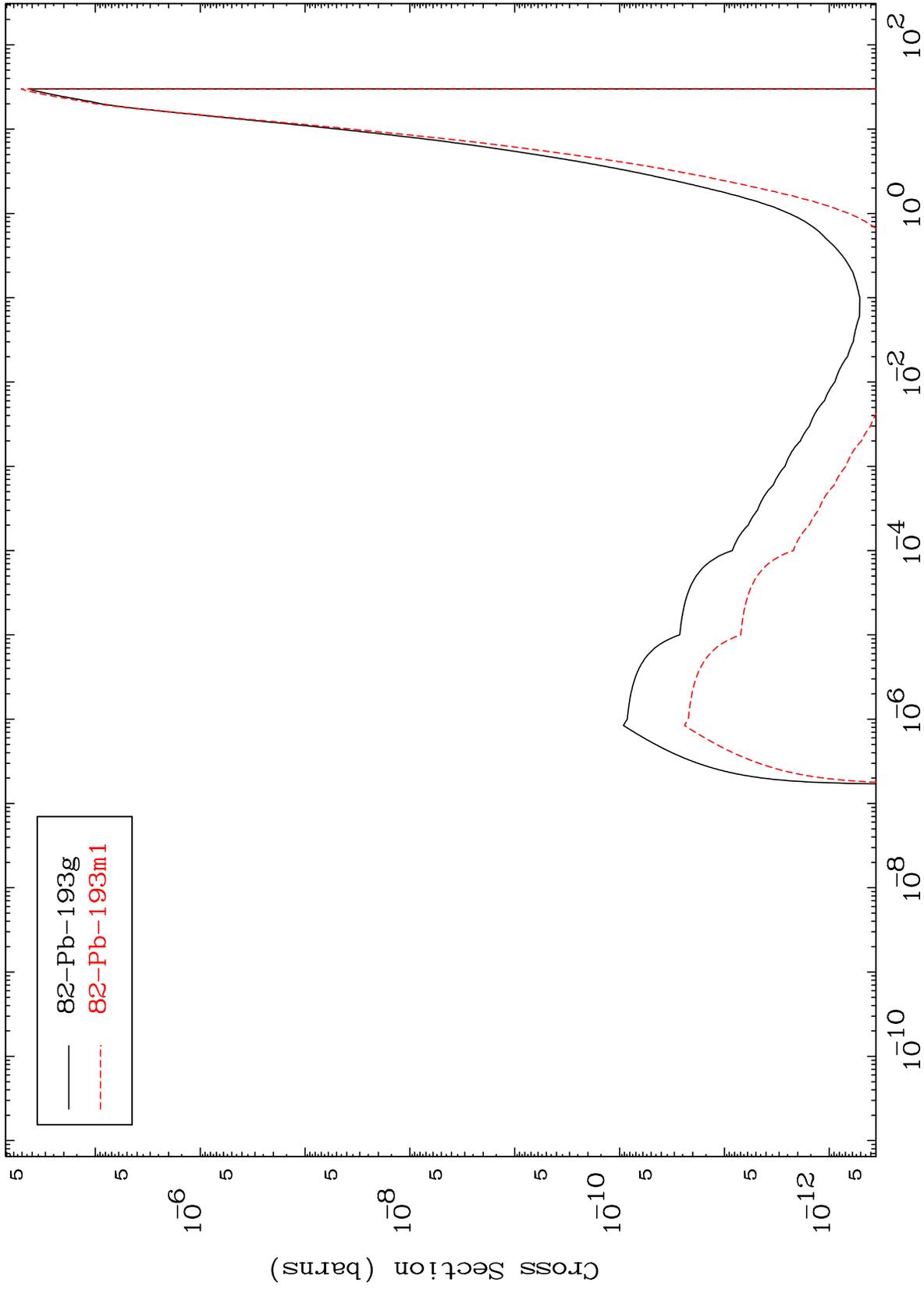
86-Rn-200

MAT 8592

(n,2α)

86-Rn-200

Radionuclide Production Cross Section

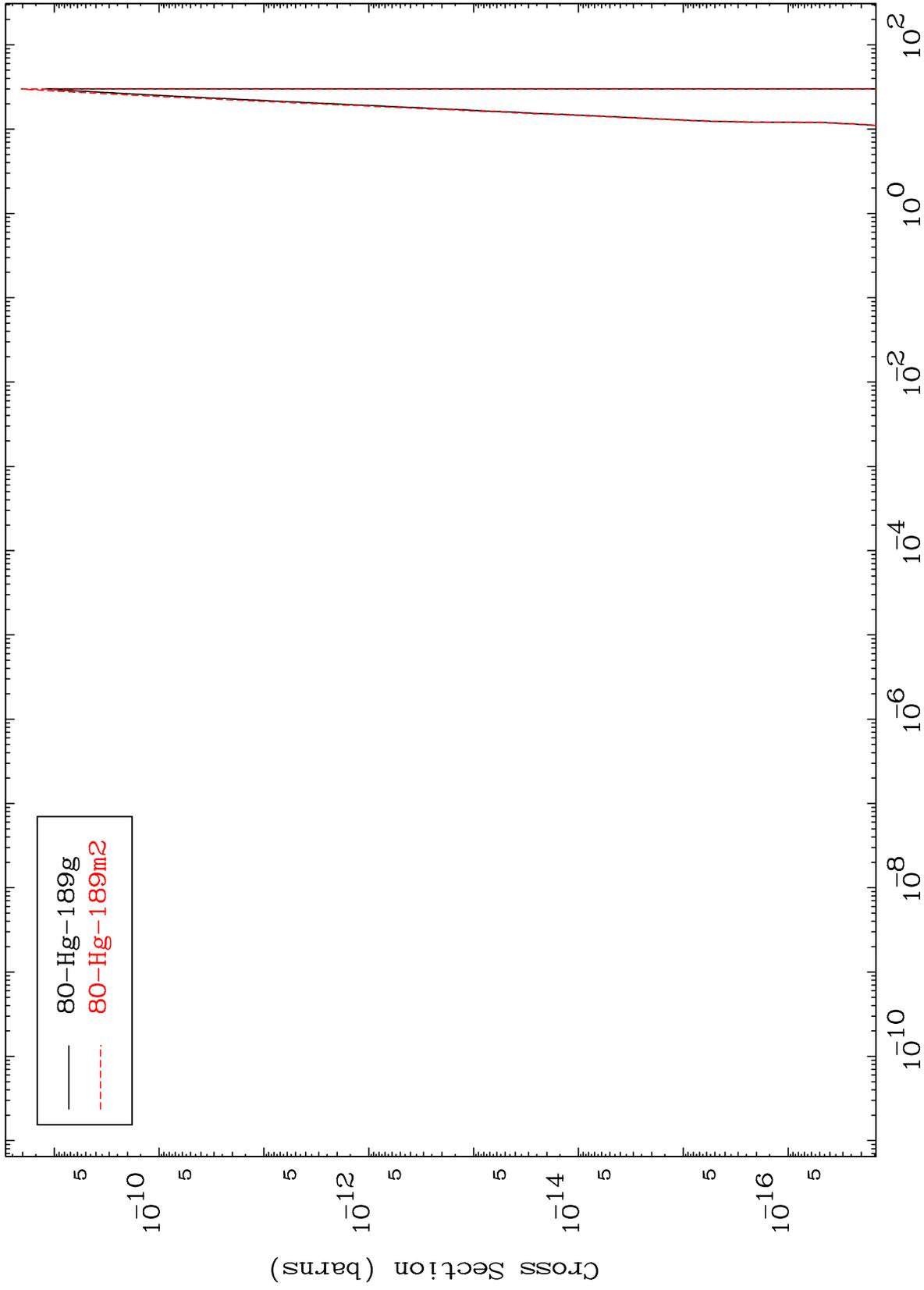


MAT 8592

(n,3α)

86-Rn-200

Radionuclide Production Cross Section

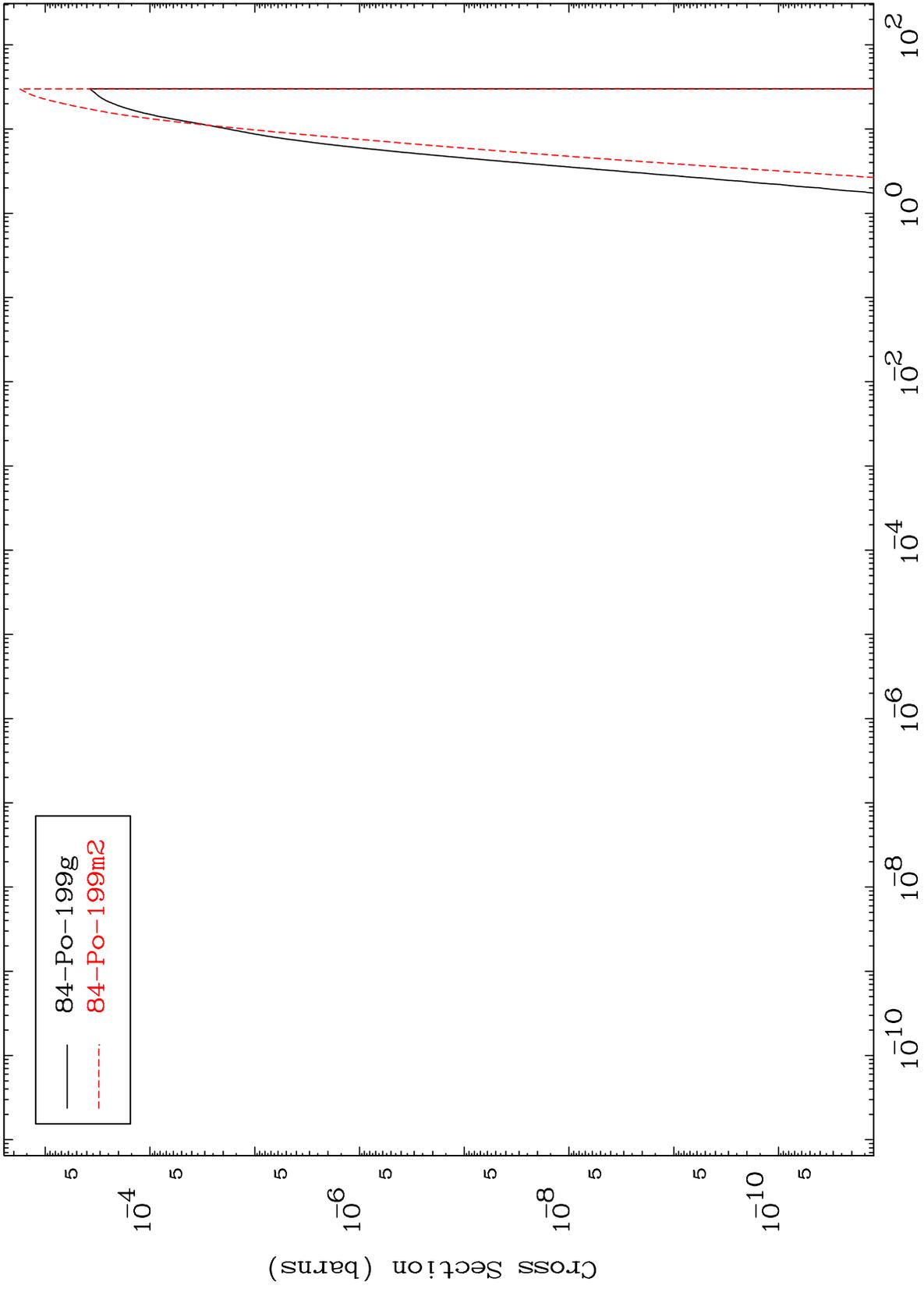


86-Rn-200

MAT 8592

Radionuclide Production Cross Section
(n,2p)

86-Rn-200



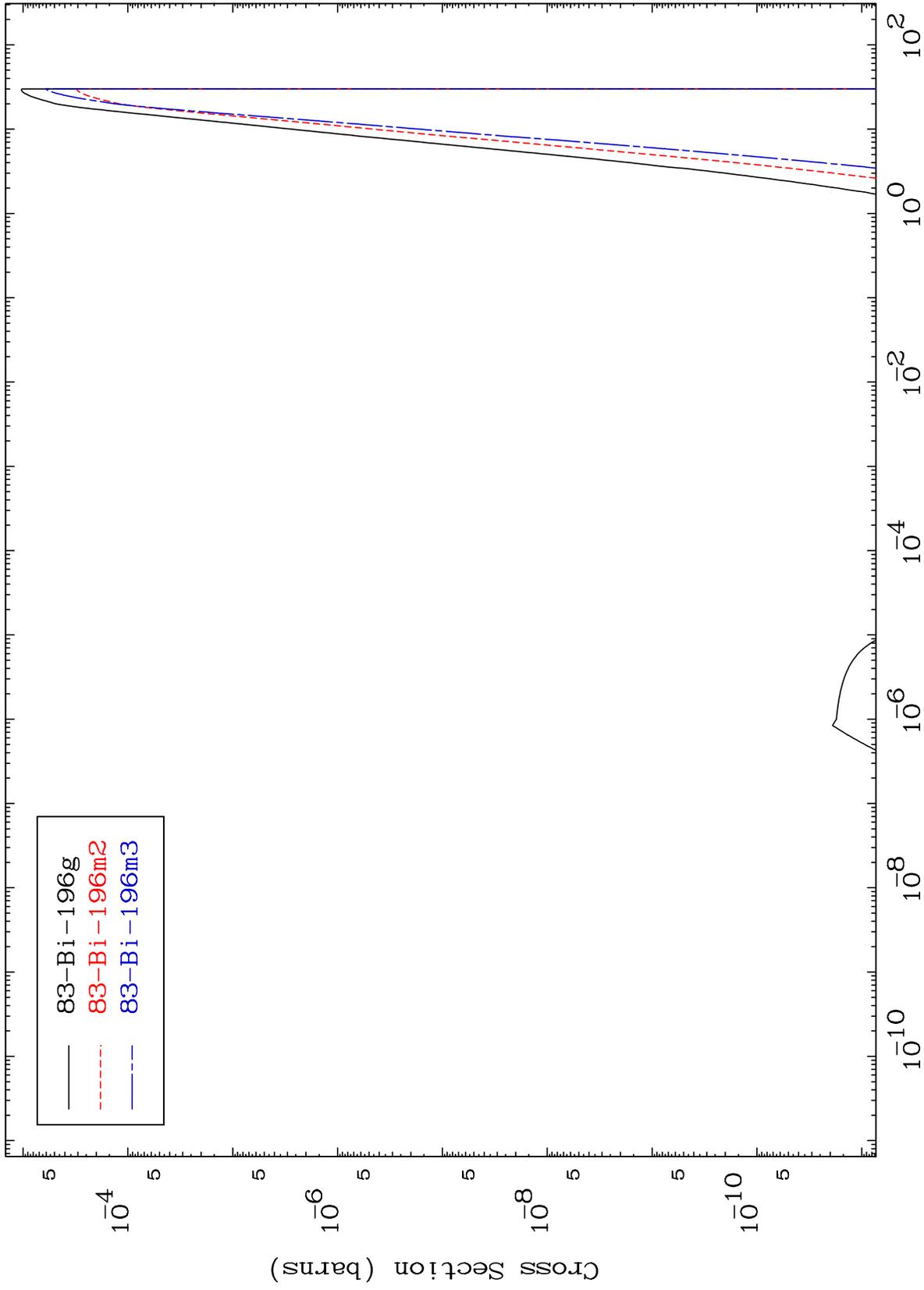
84-Po-199g
84-Po-199m2

MAT 8592

$(n,p) \alpha$

$^{86}\text{Rn-200}$

Radionuclide Production Cross Section



46

Incident Energy (MeV)

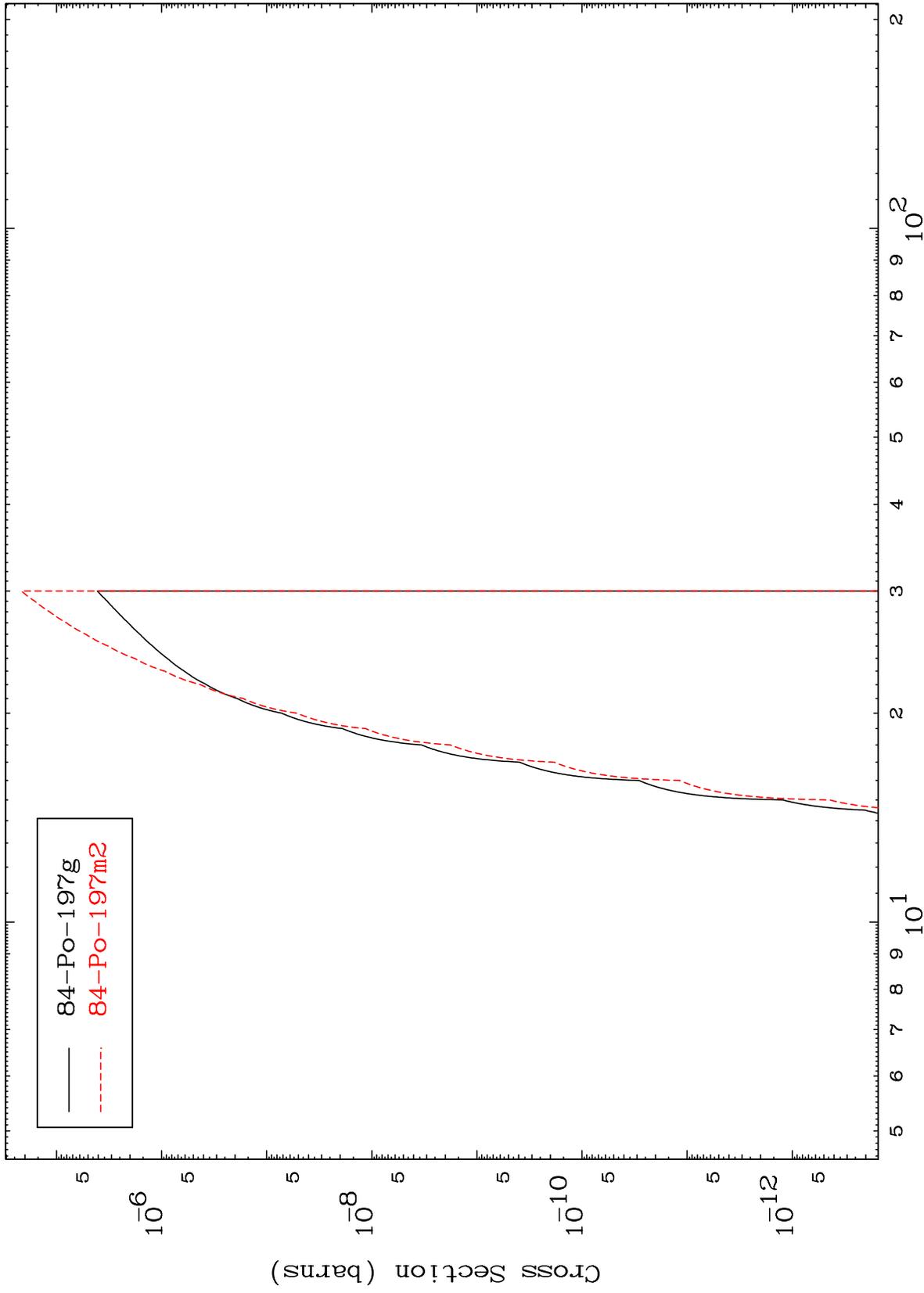
$^{86}\text{Rn-200}$

MAT 8592

(n,p) t

86-Rn-200

Radionuclide Production Cross Section



47

Incident Energy (MeV)

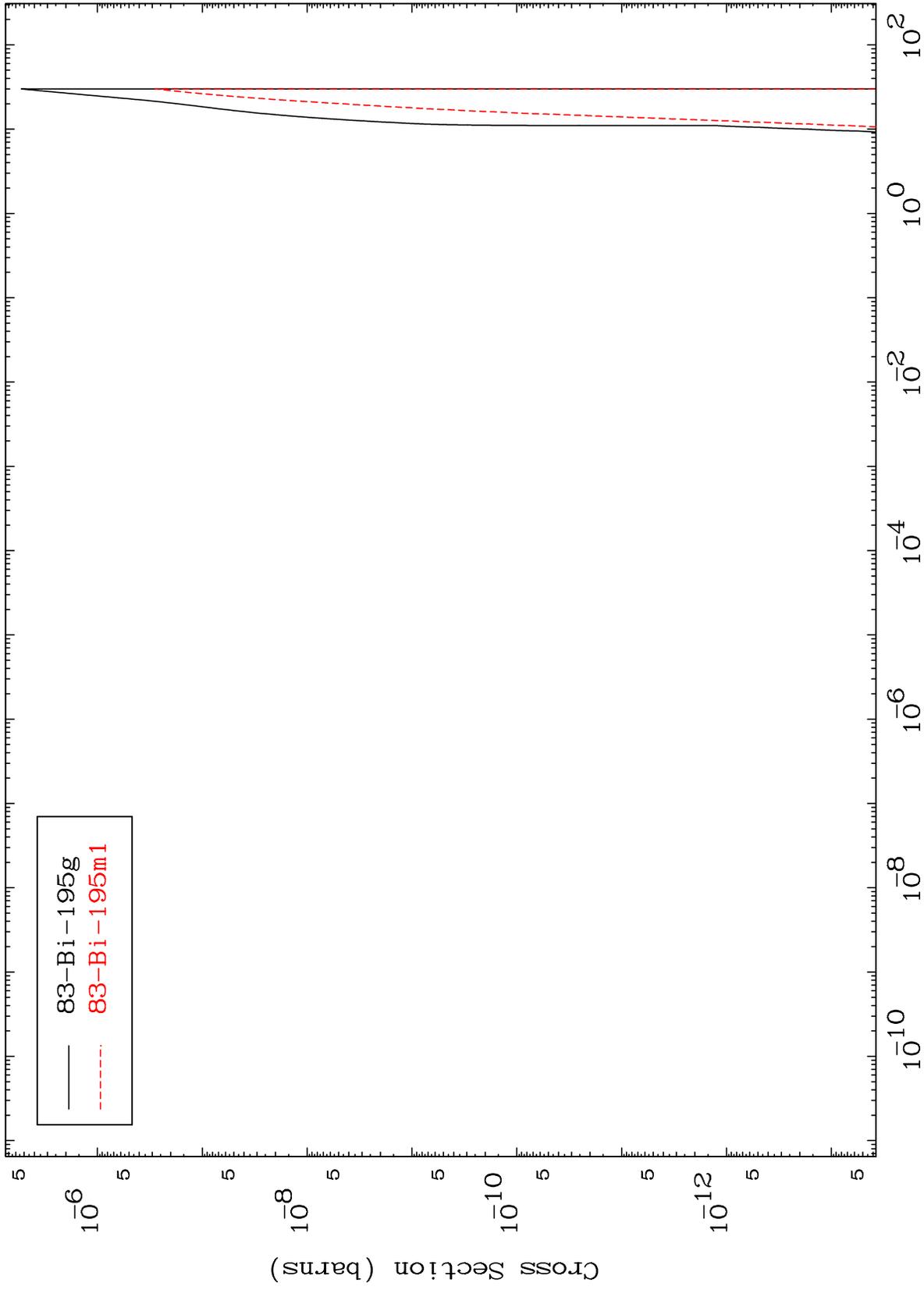
86-Rn-200

MAT 8592

$(n, d) \alpha$

86-Rn-200

Radionuclide Production Cross Section



48

Incident Energy (MeV)

86-Rn-200