

Program EVALPLOT
(Version 2021-1)

by

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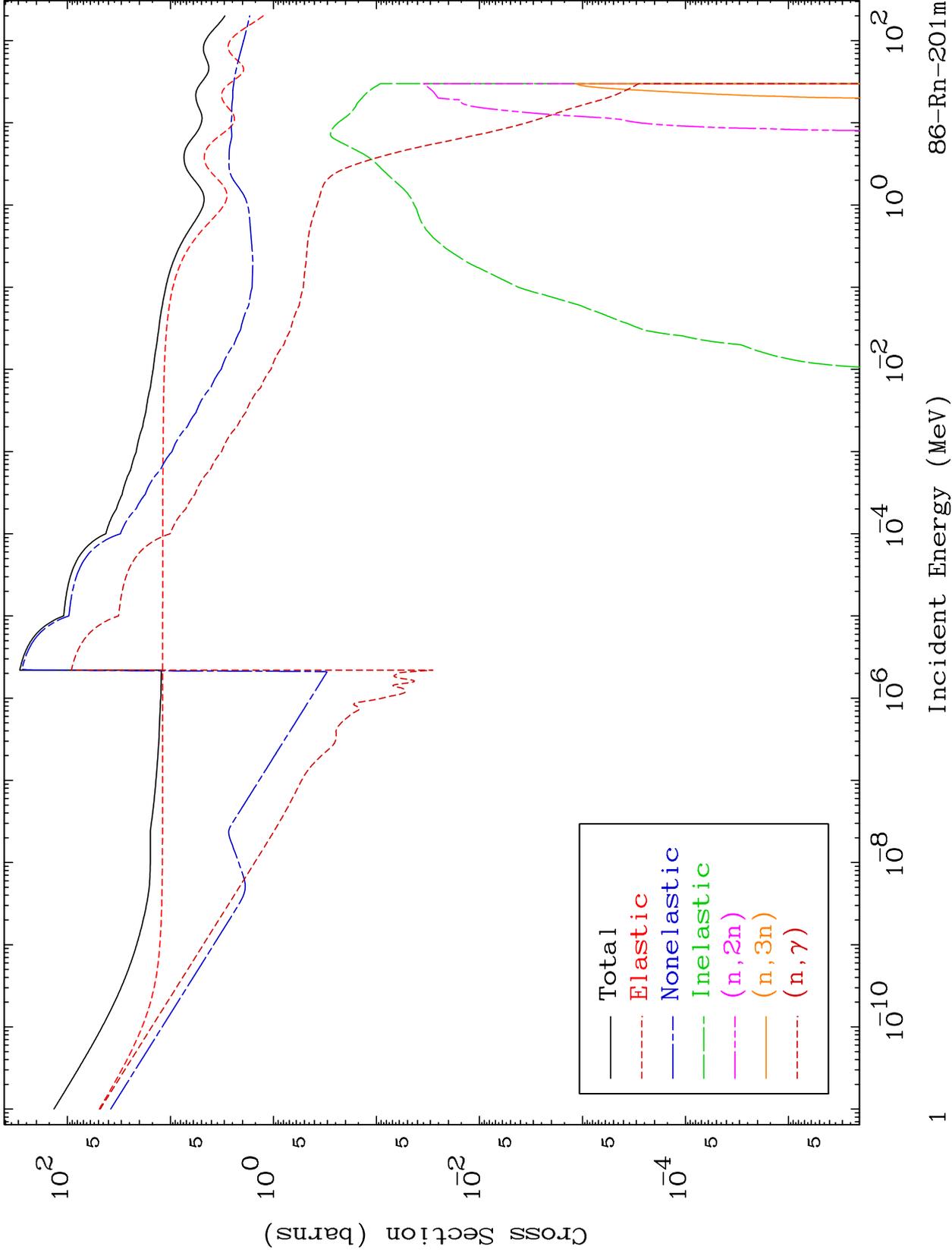
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Press Mouse Button to Start

MAT 8596

Neutron Major
293 Kelvin Cross Sections

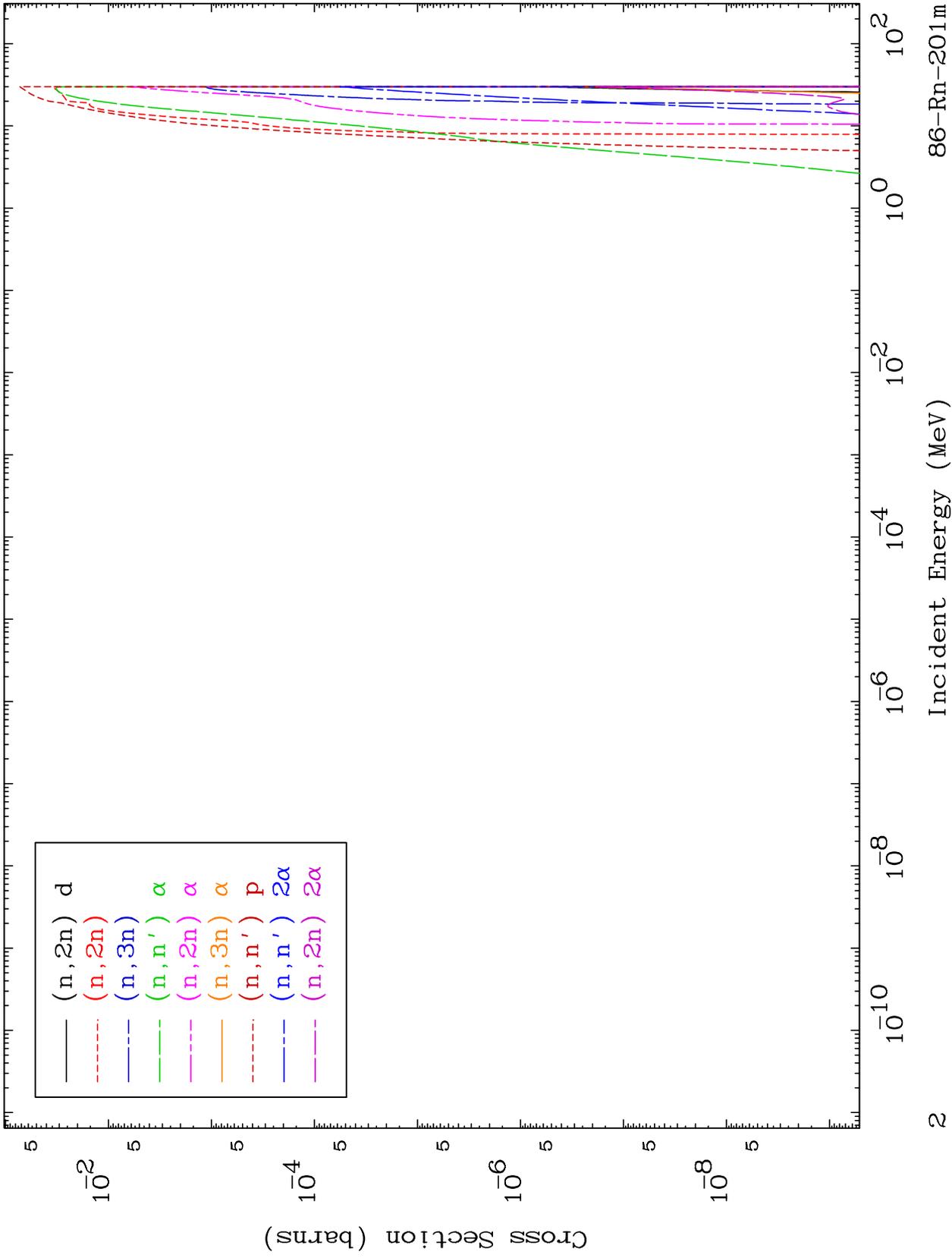
86-Rn-201m



MAT 8596

Neutron Absorption
293 Kelvin Cross Sections

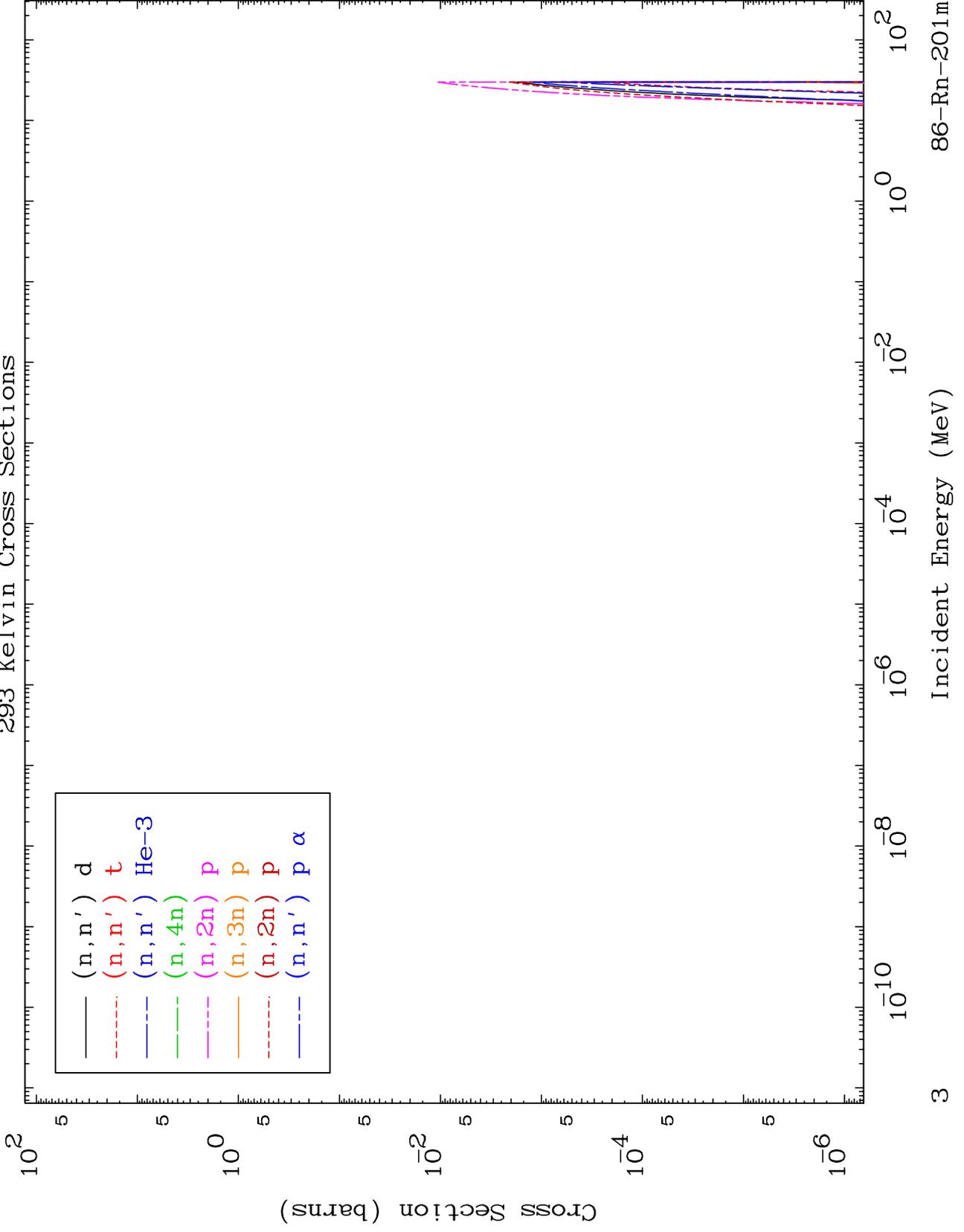
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MAT 8596

Neutron Absorption
293 Kelvin Cross Sections

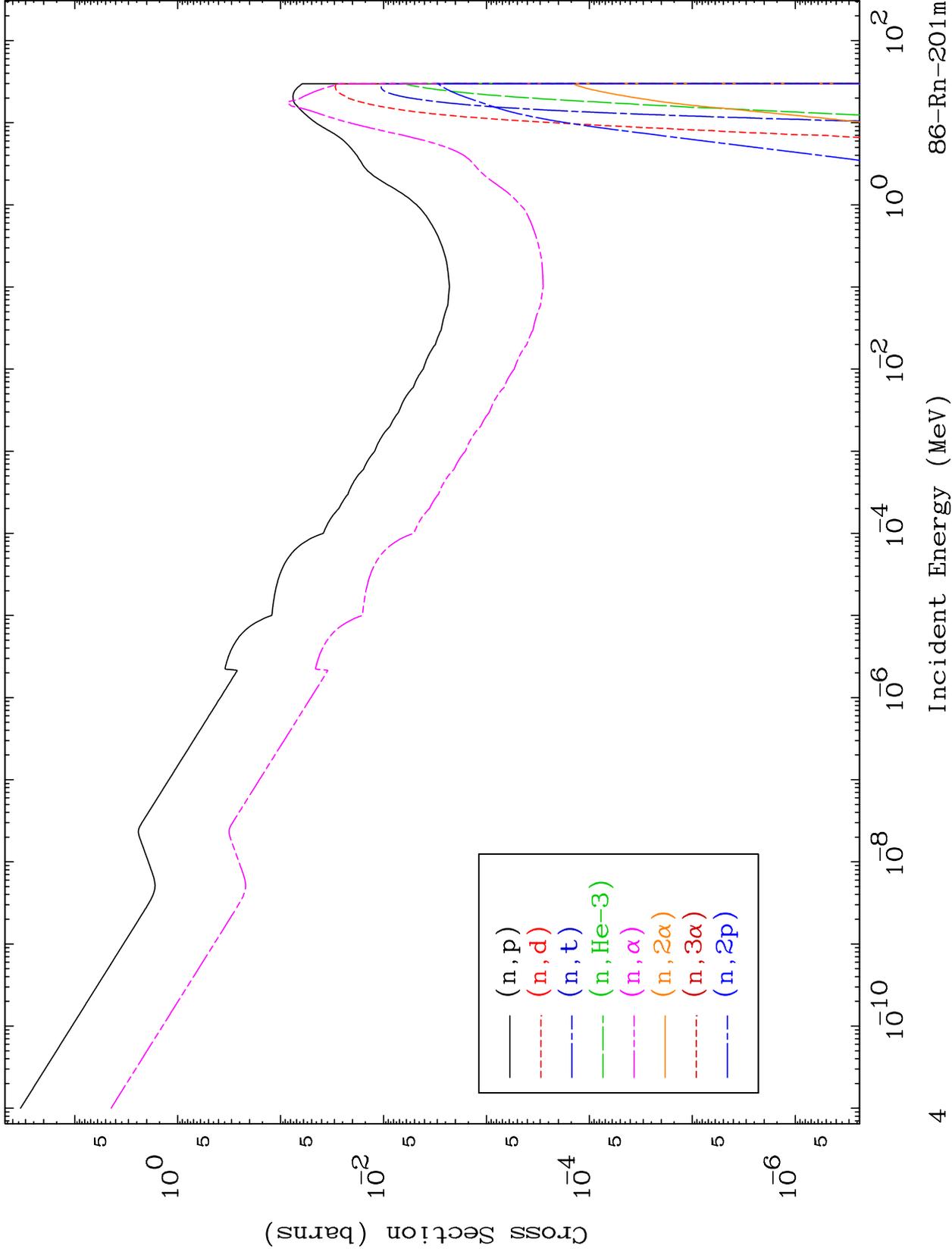
86-Rn-201m



MAT 8596

Neutron Absorption
293 Kelvin Cross Sections

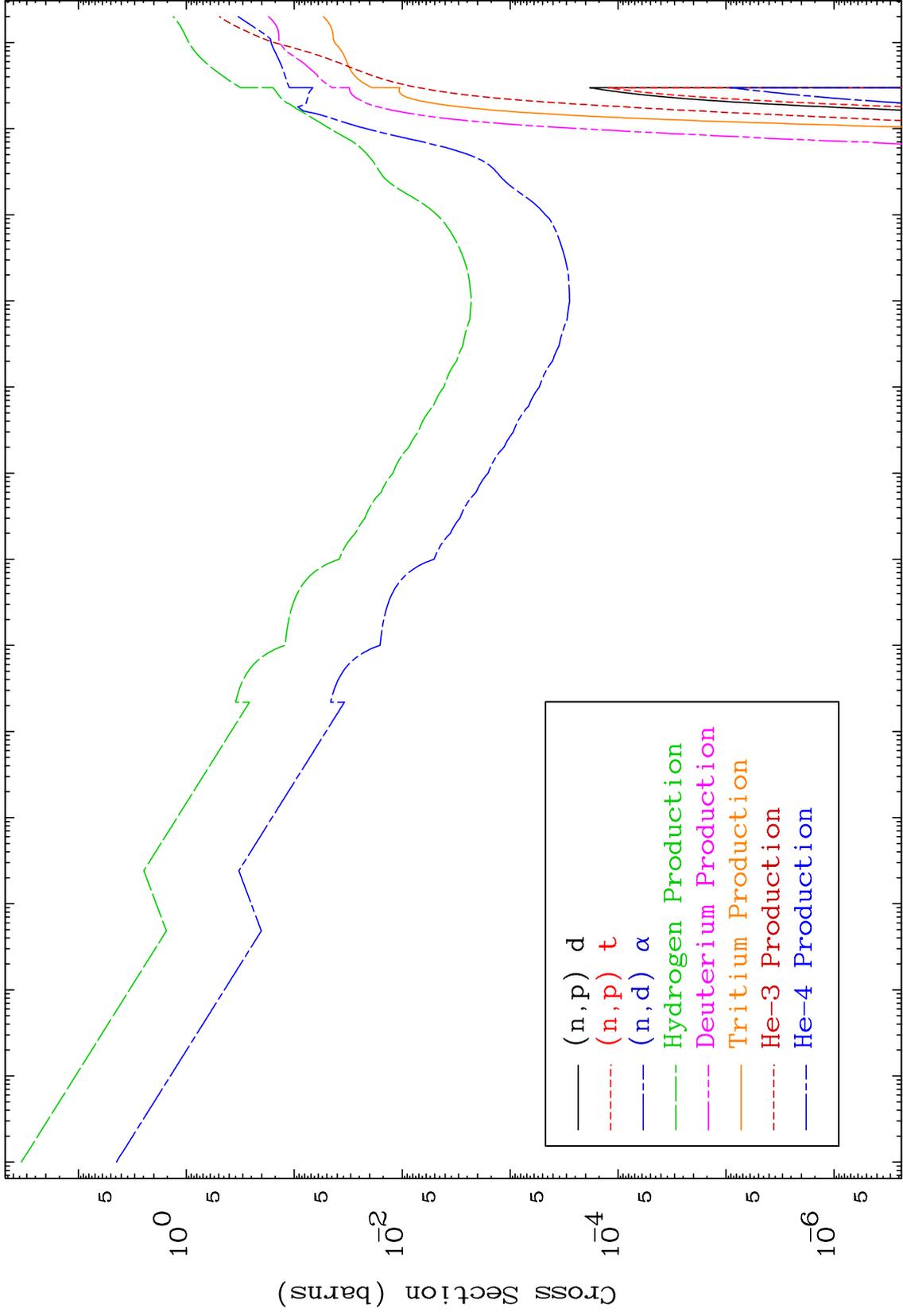
86-Rn-201m



MAT 8596

Neutron Absorption
293 Kelvin Cross Sections

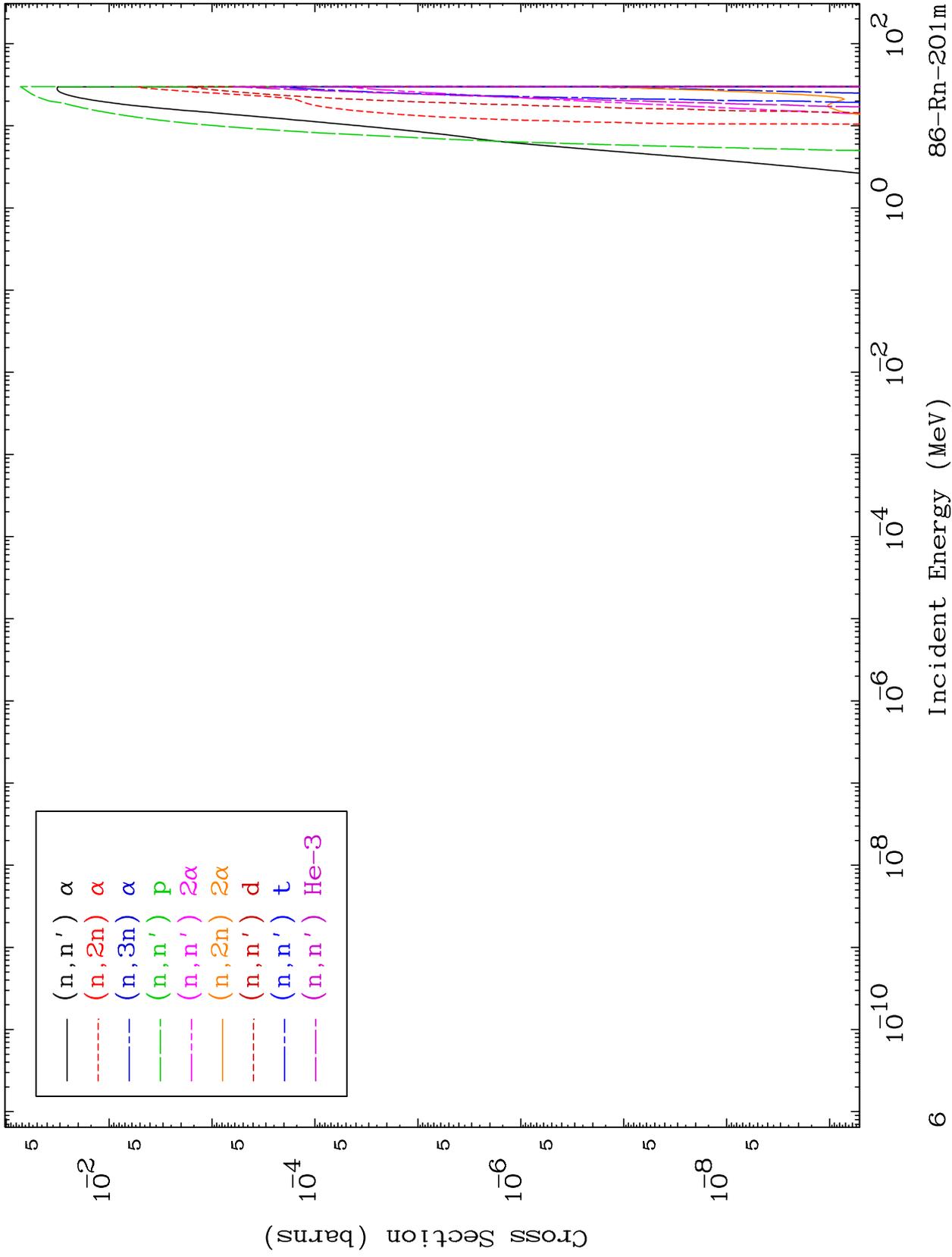
86-Rn-201m



MAT 8596

Charged Particle
293 Kelvin Cross Sections

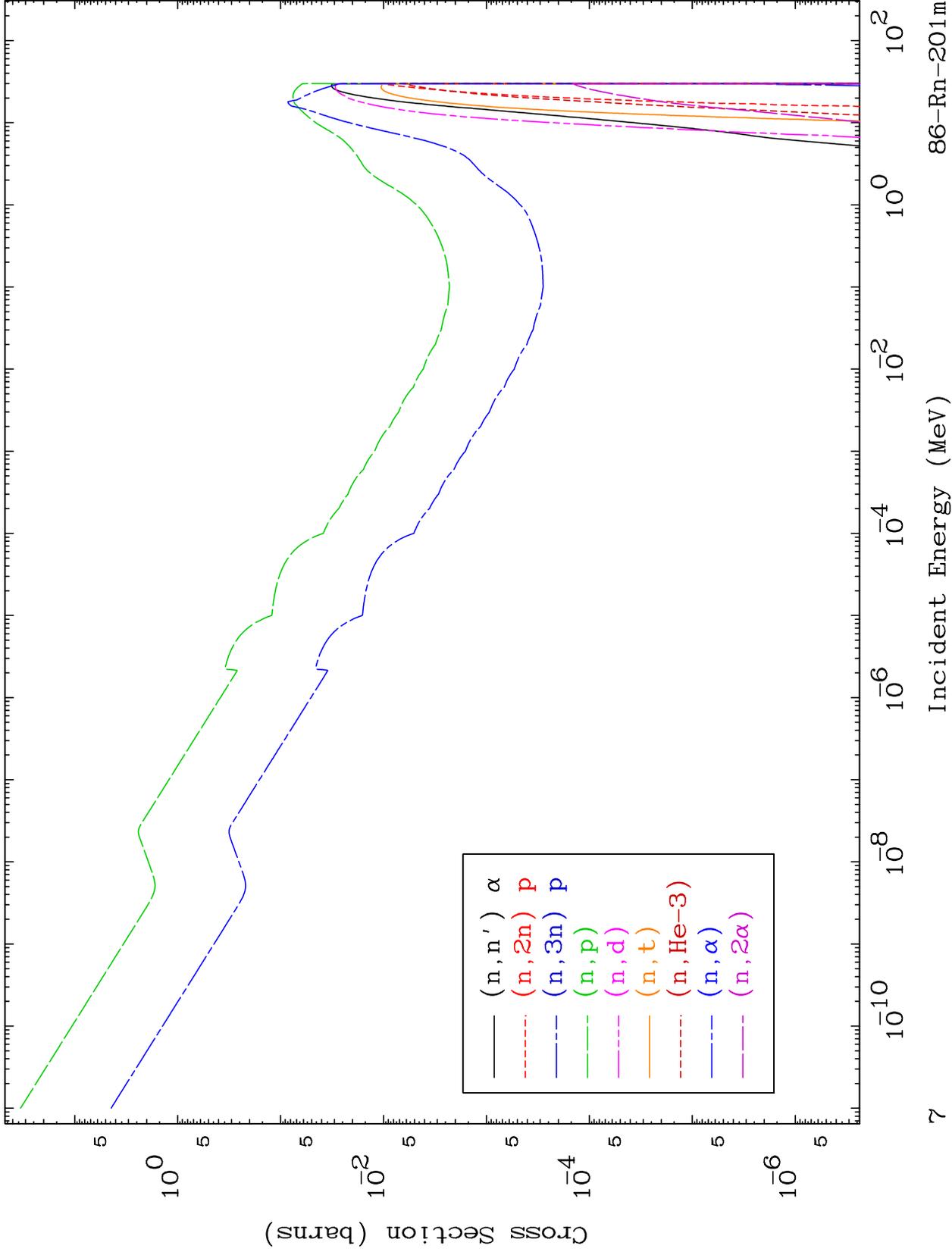
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MAT 8596

Charged Particle
293 Kelvin Cross Sections

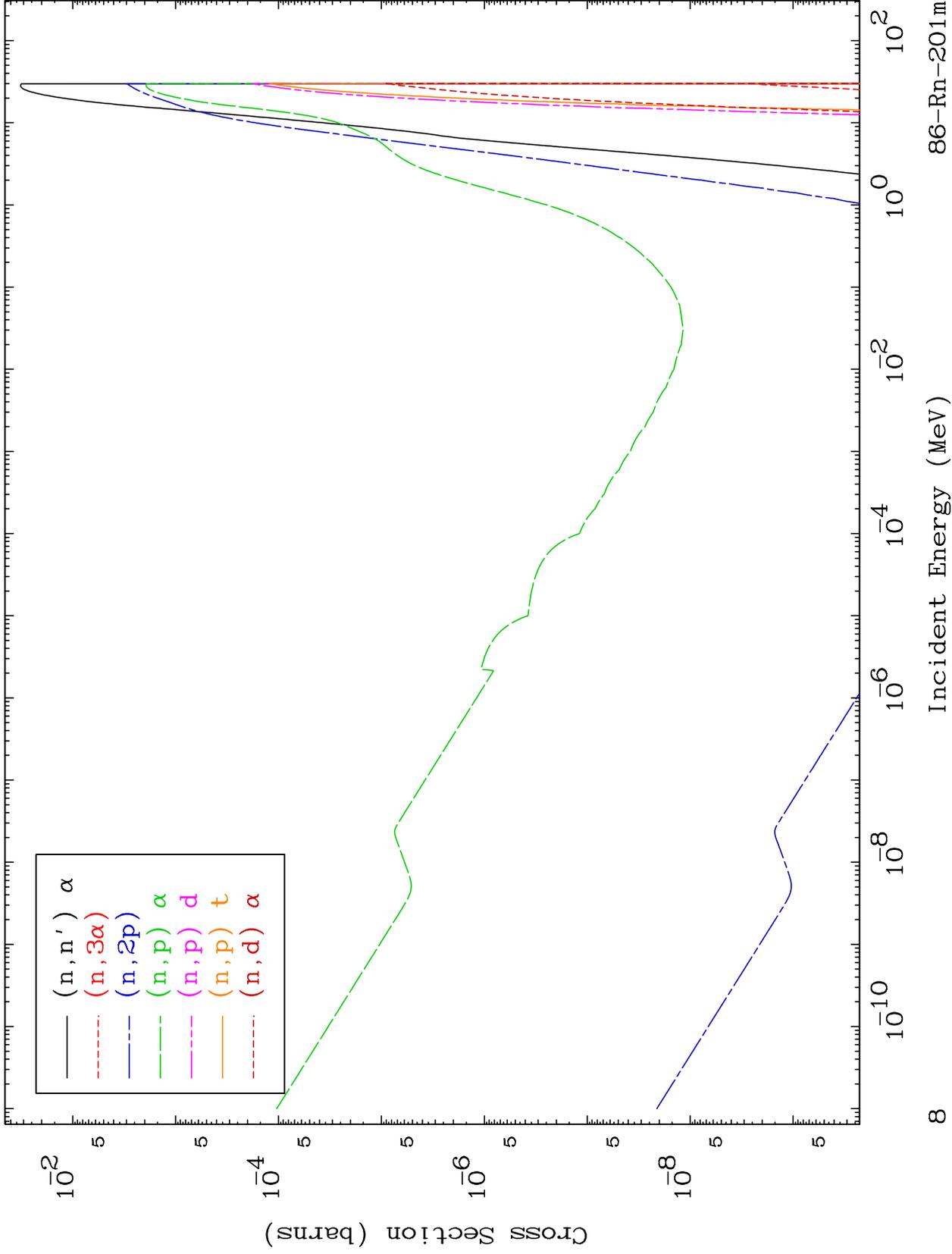
86-Rn-201m



MAT 8596

Charged Particle
293 Kelvin Cross Sections

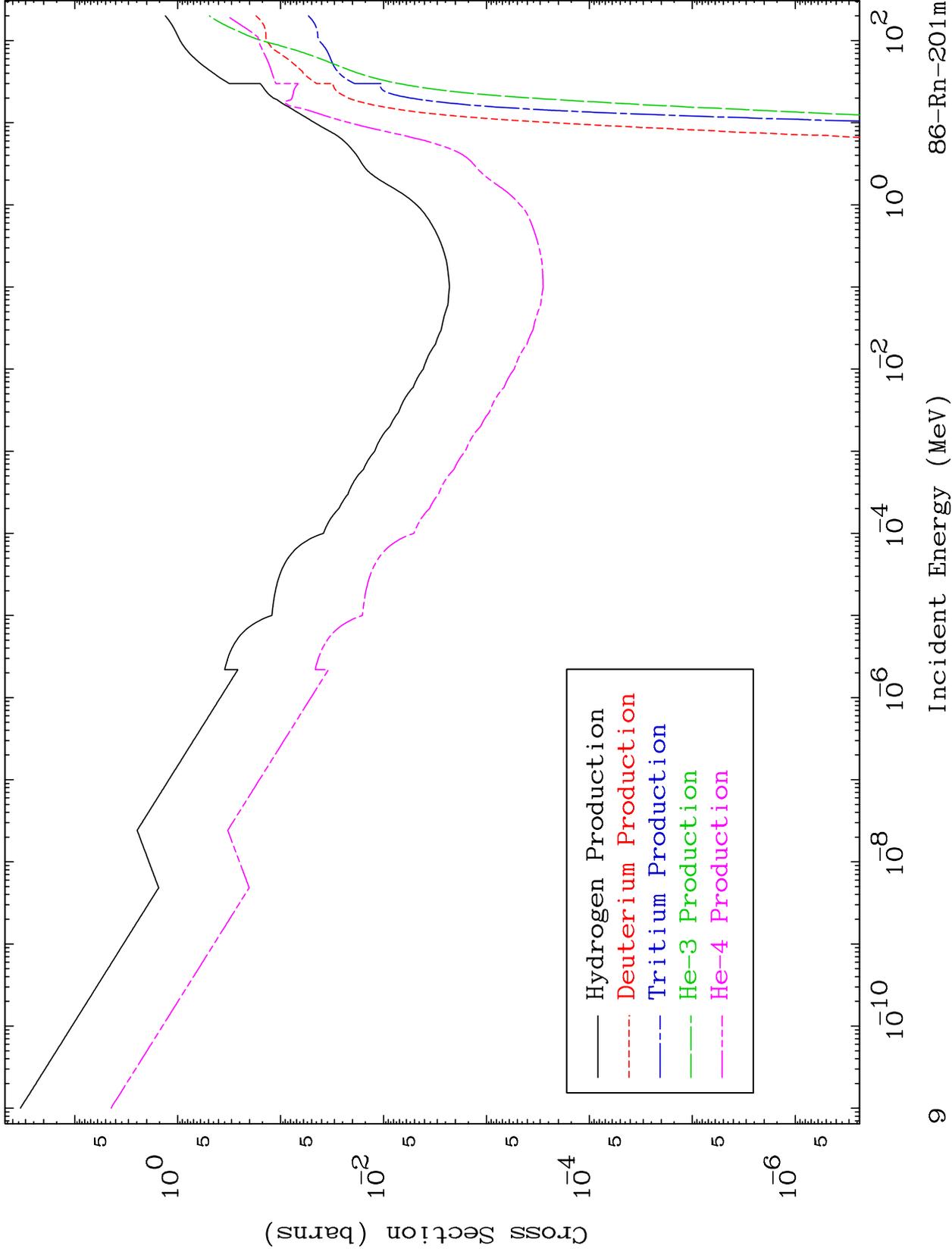
86-Rn-201m



MAT 8596

Particle Production
293 Kelvin Cross Sections

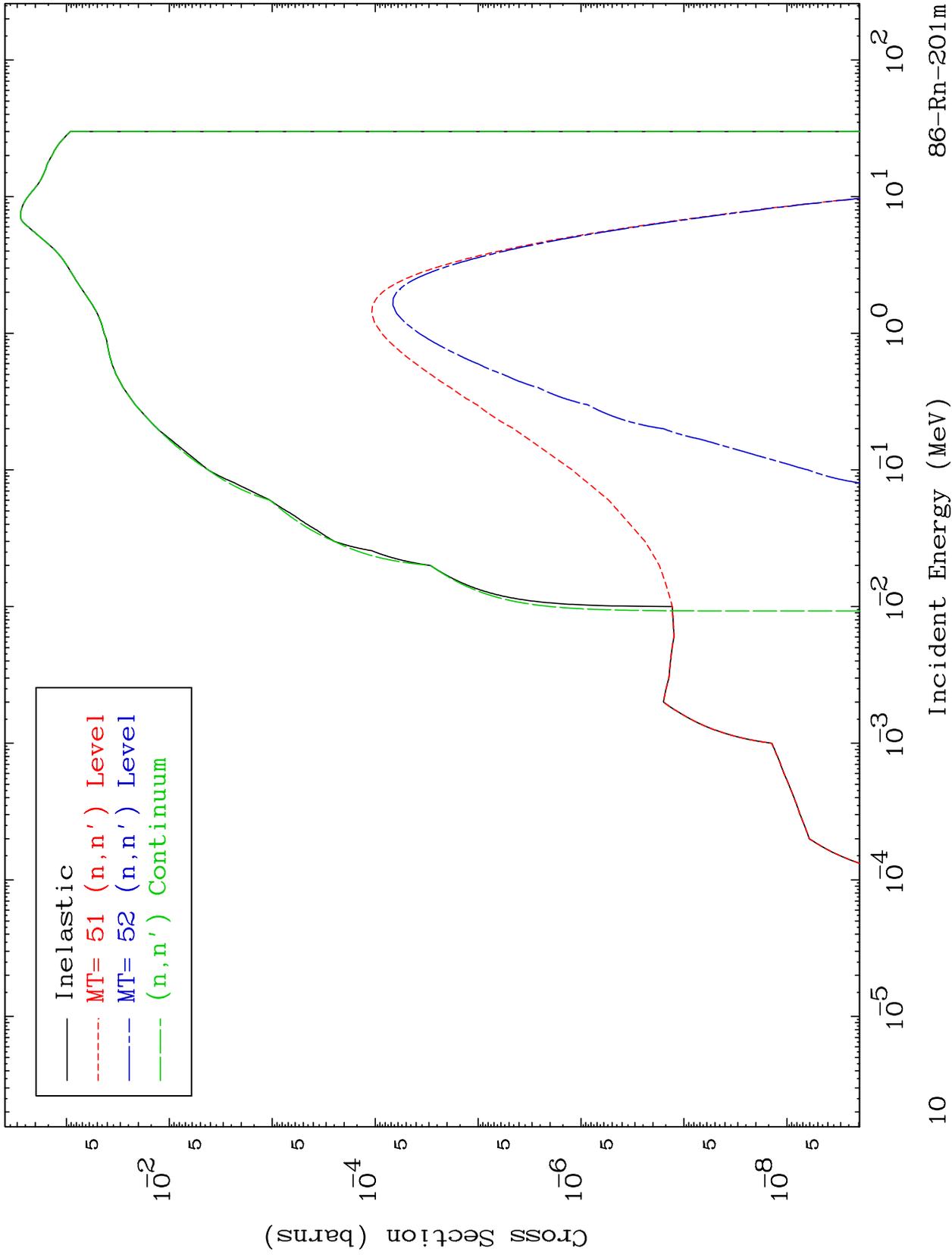
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MAT 8596

(n,n') Levels
293 Kelvin Cross Sections

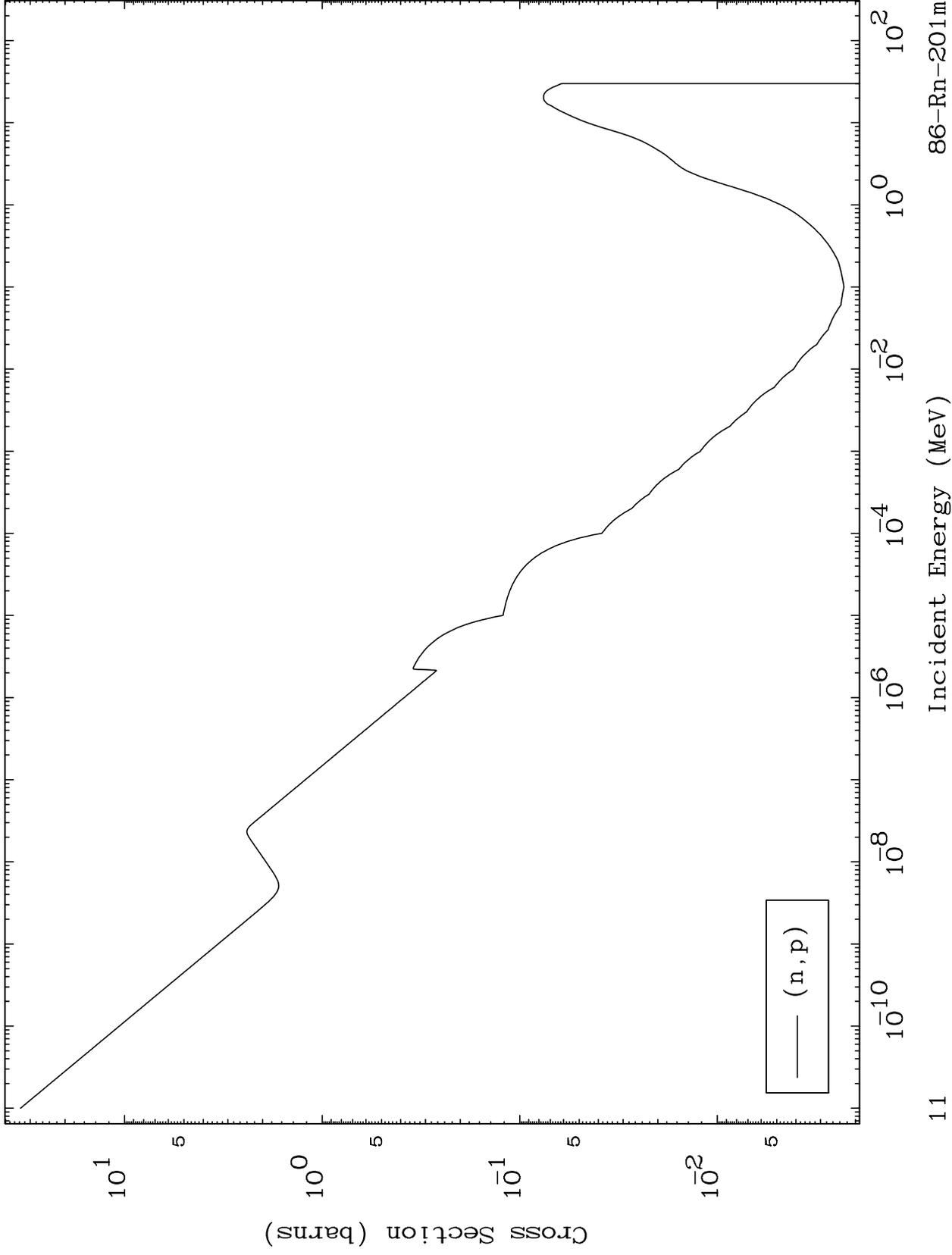
86-Rn-201m



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(n,p) Levels
293 Kelvin Cross Sections

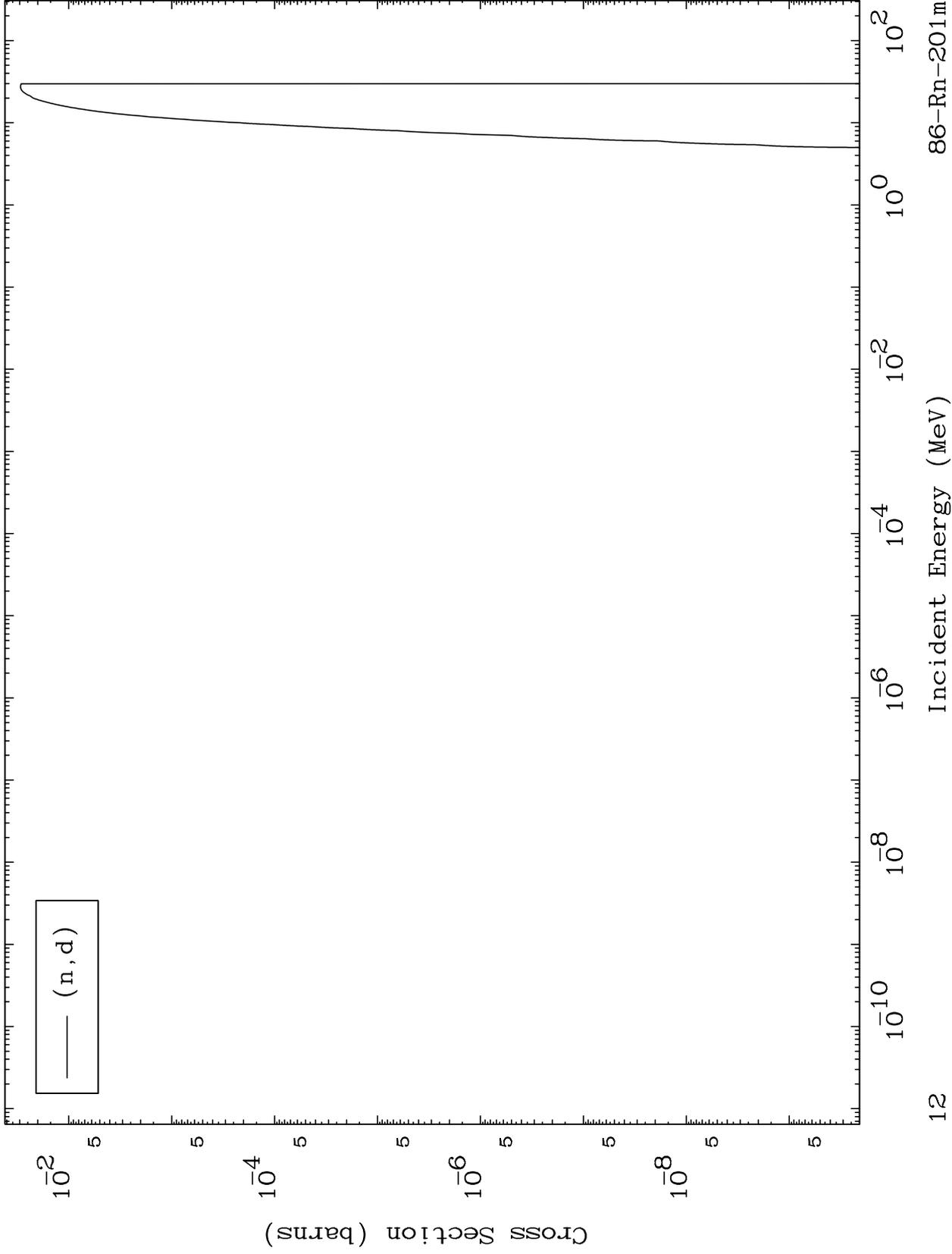
86-Rn-201m



MAT 8596

(n,d) Levels
293 Kelvin Cross Sections

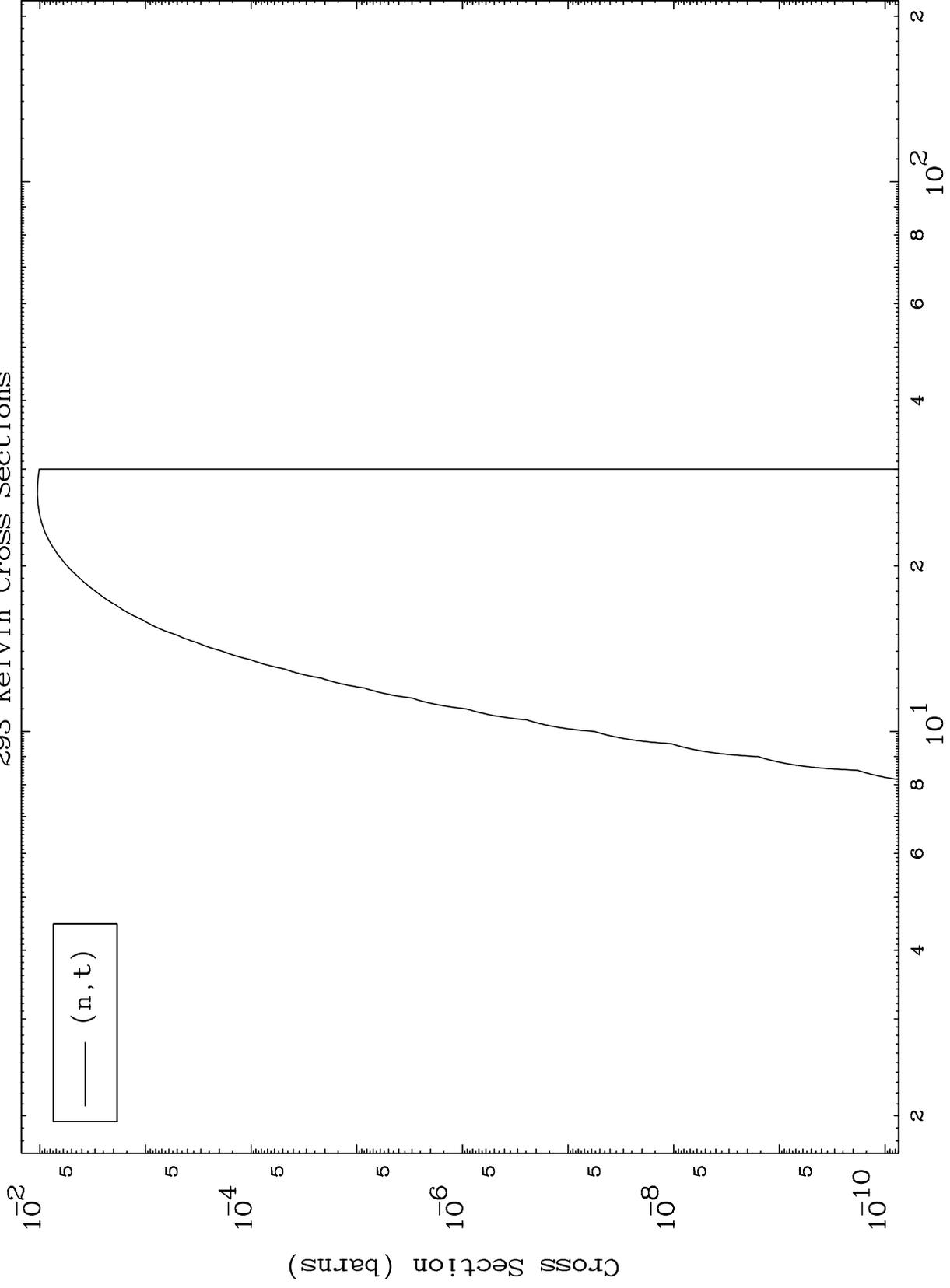
86-Rn-201m



MAT 8596

(n,t) Levels
293 Kelvin Cross Sections

86-Rn-201m



(n,t)

13

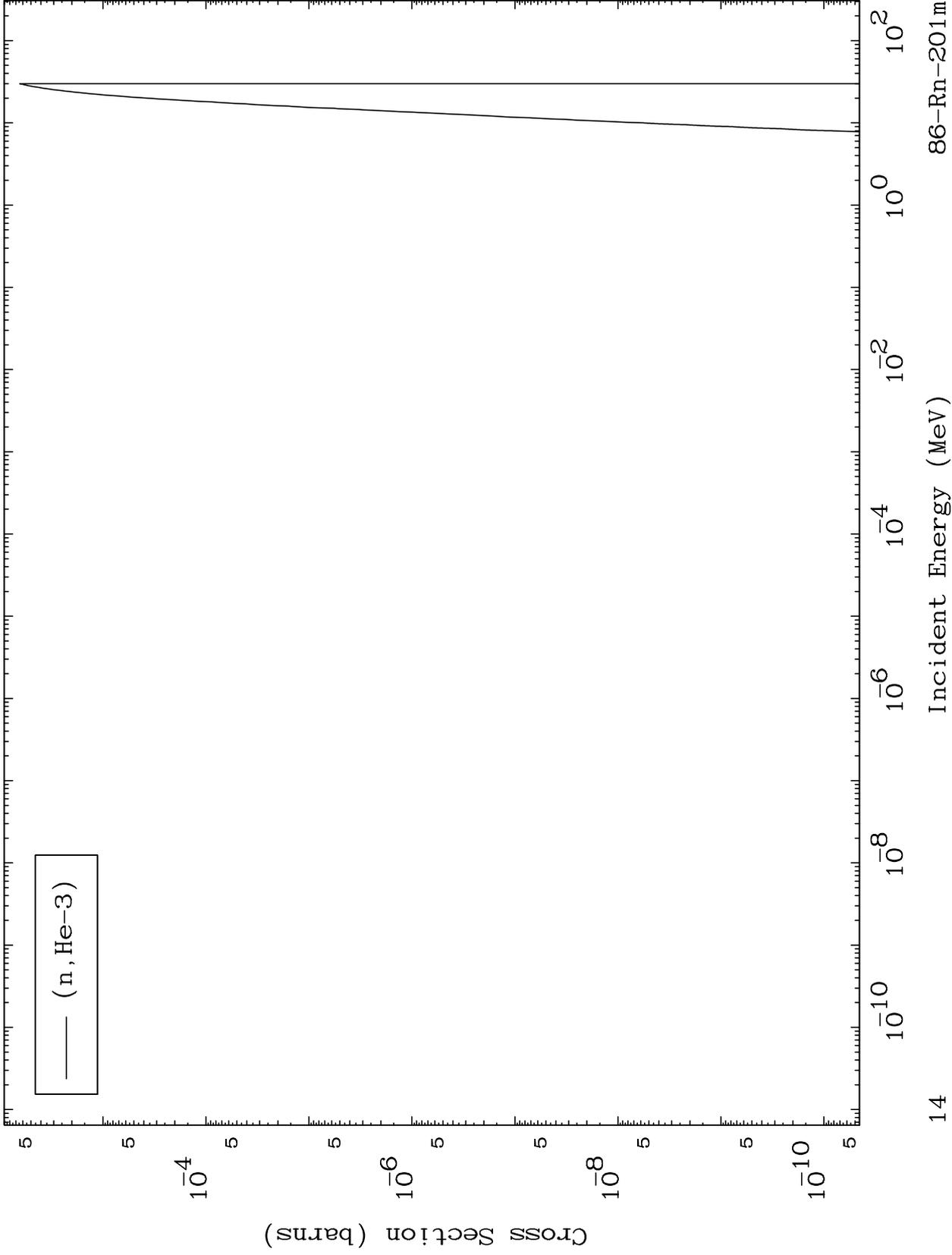
Incident Energy (MeV)

86-Rn-201m

MAT 8596

(n,He3) Levels
293 Kelvin Cross Sections

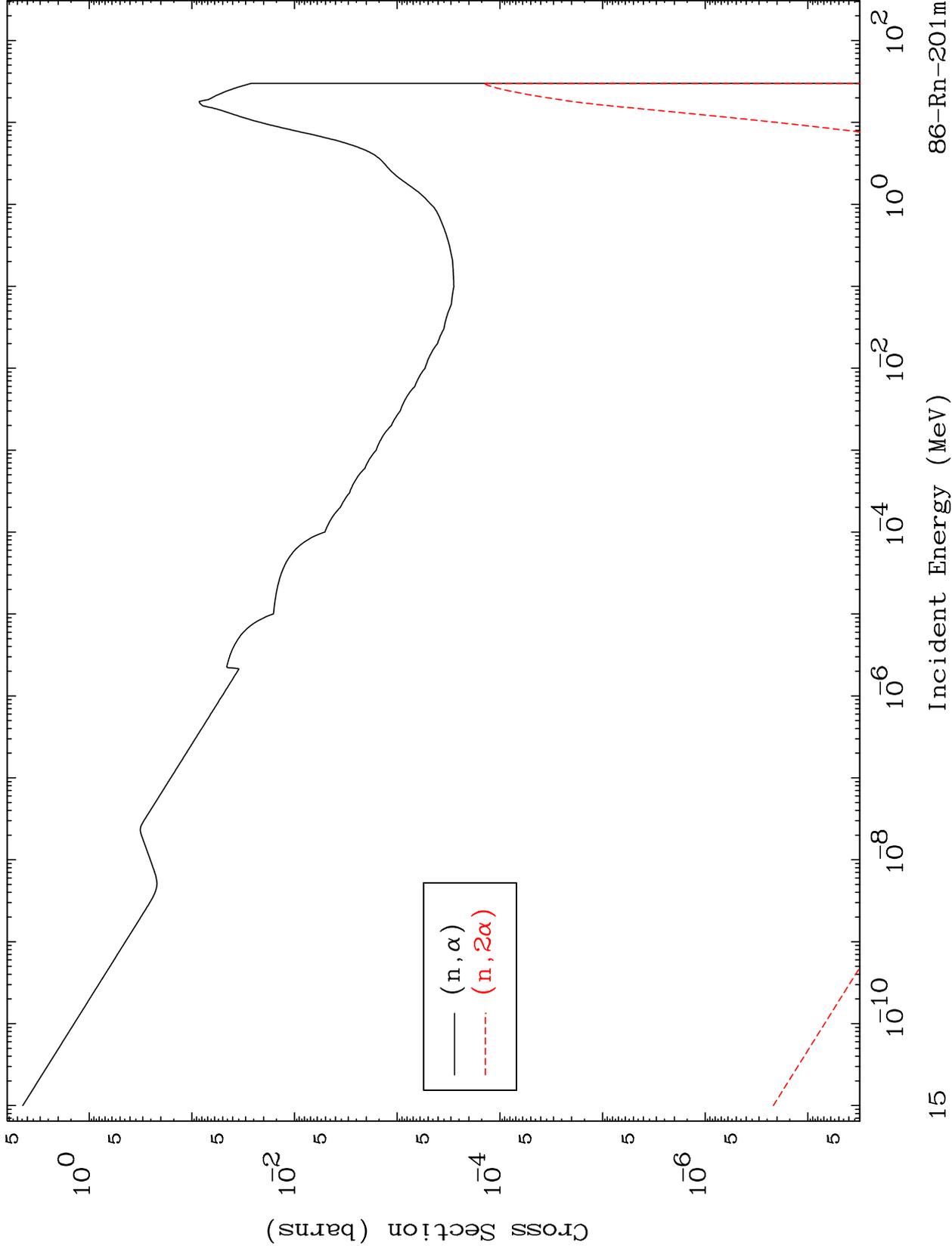
86-Rn-201m



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(n, α) Levels
293 Kelvin Cross Sections

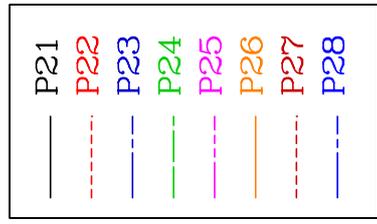
86-Rn-201m



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Elastic Legendre Coefficients

86-Rn-201m



$\times 10^{-6}$

2.0

1.5

1.0

0.5

0.0

Legendre (CM)

15

20

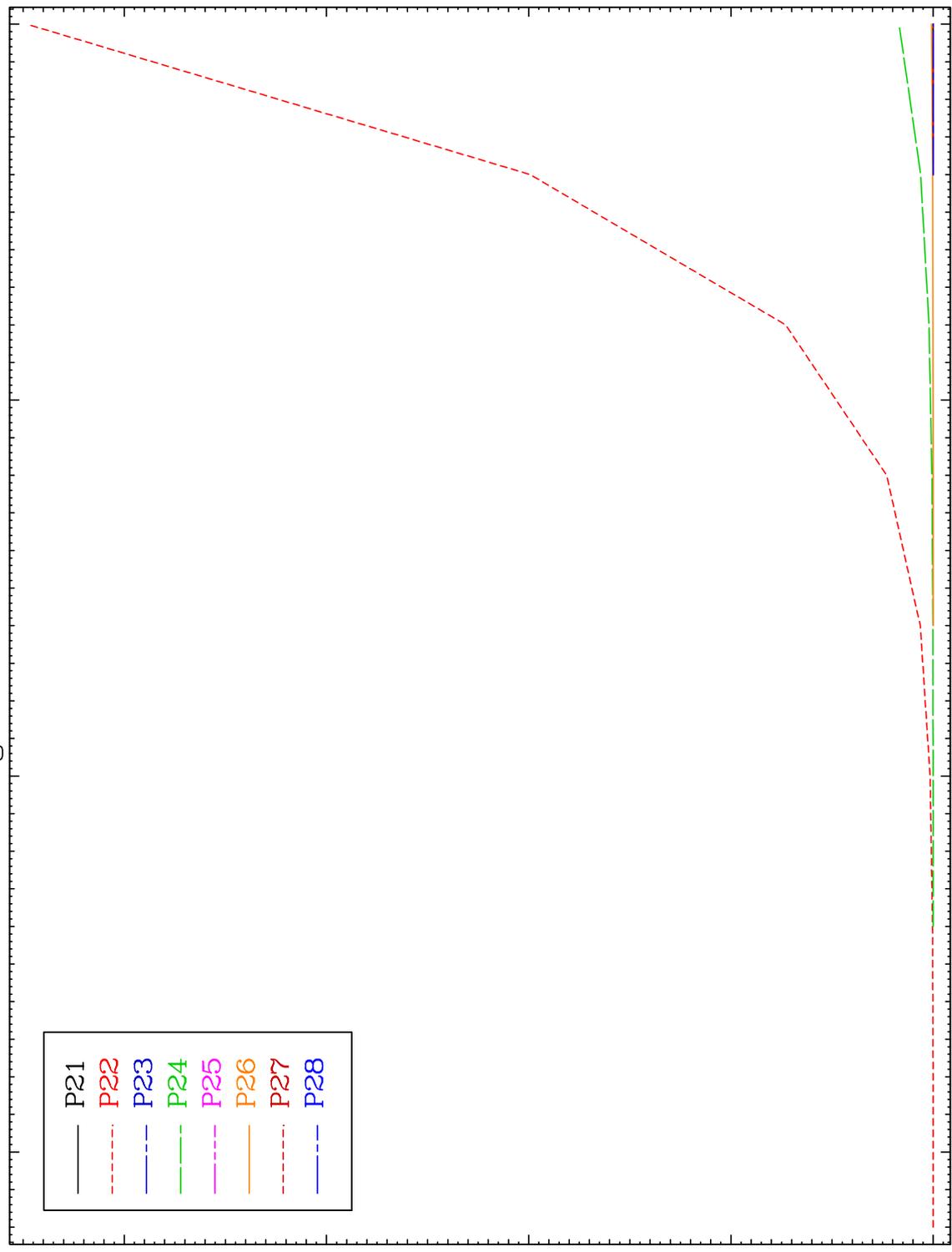
25

30

18

Incident Energy (MeV)

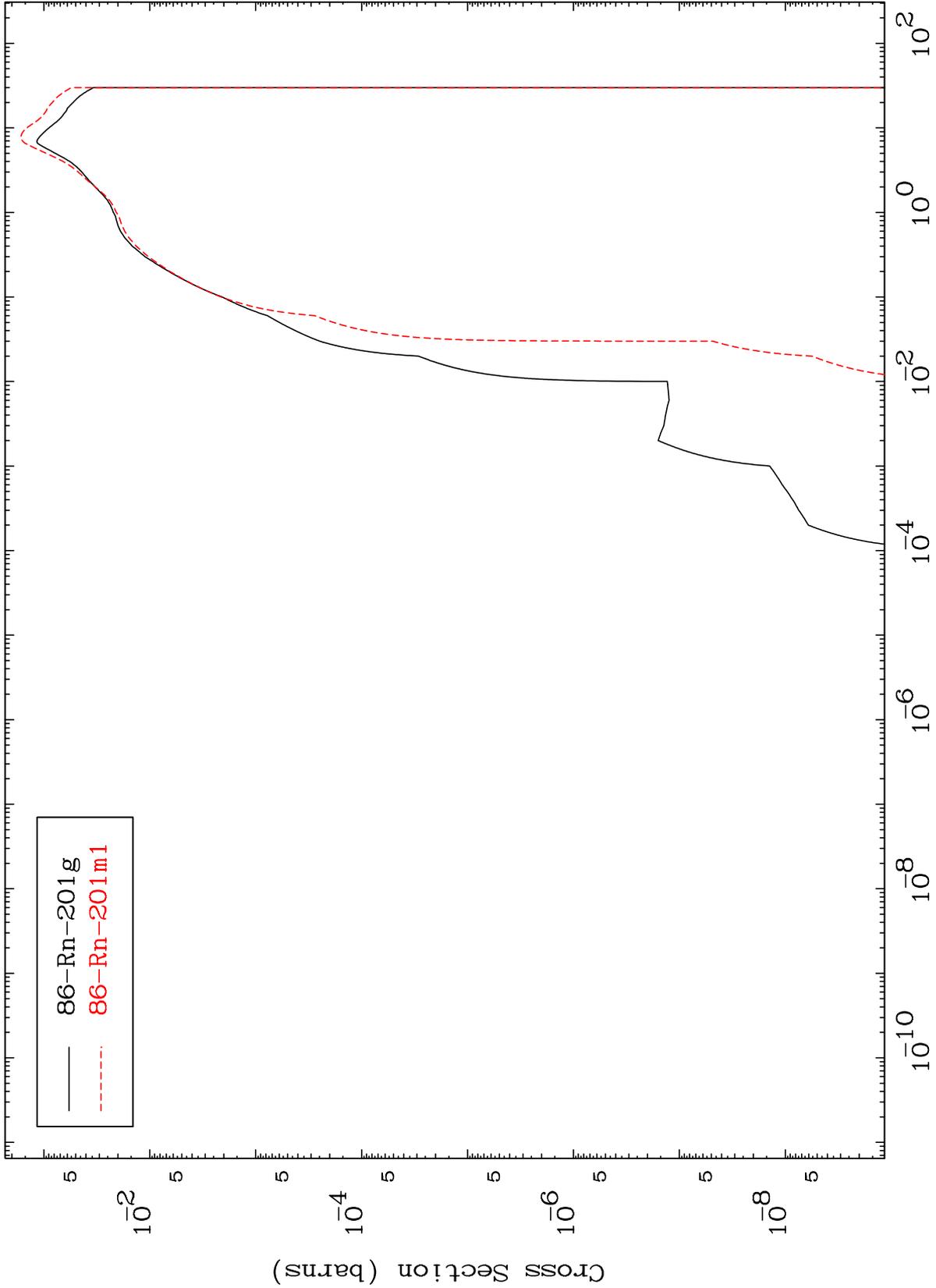
86-Rn-201m



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Inelastic
Radionuclide Production Cross Section

86-Rn-201m



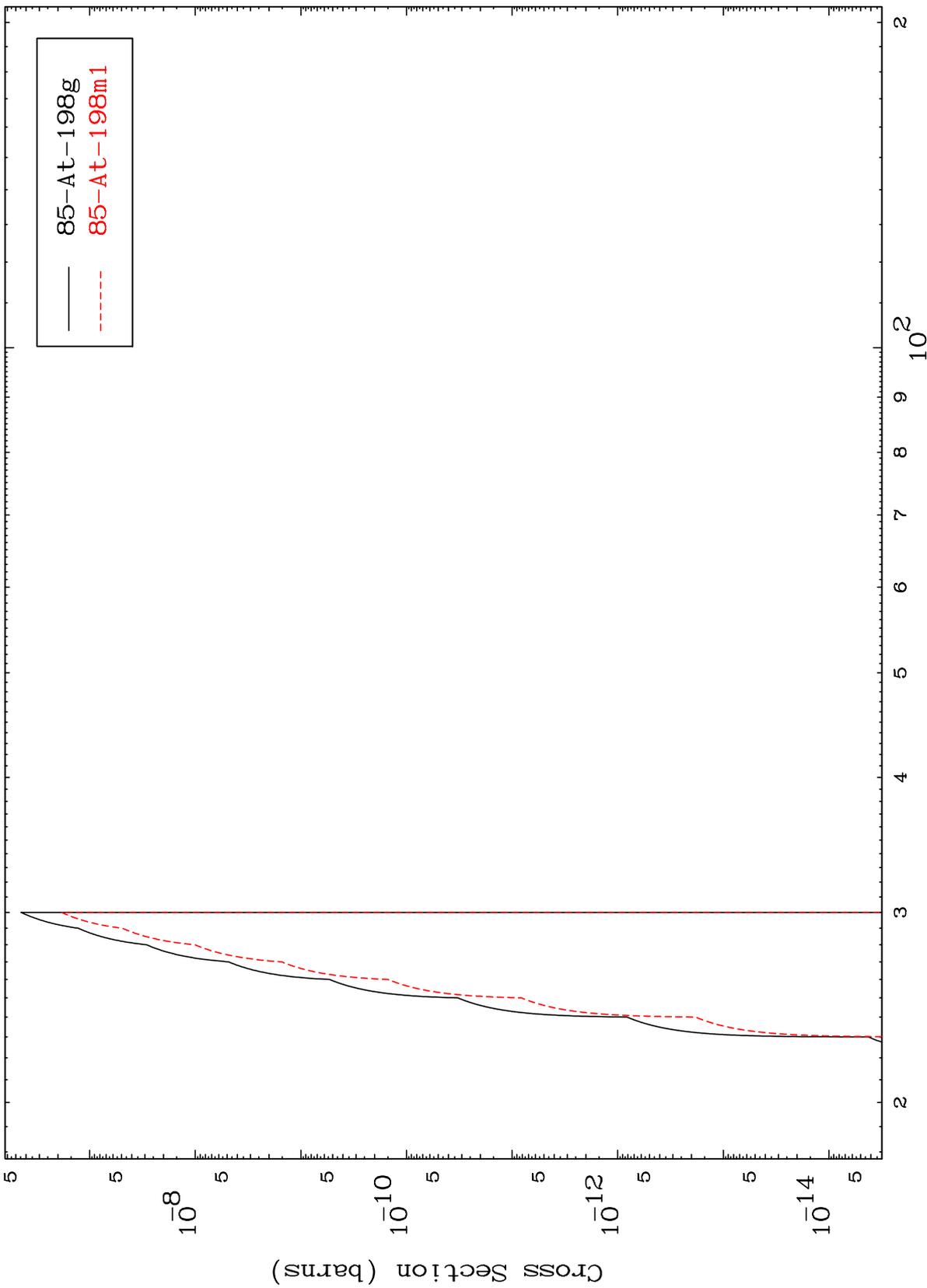
86-Rn-201m

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(n,2n) d

86-Rn-201m

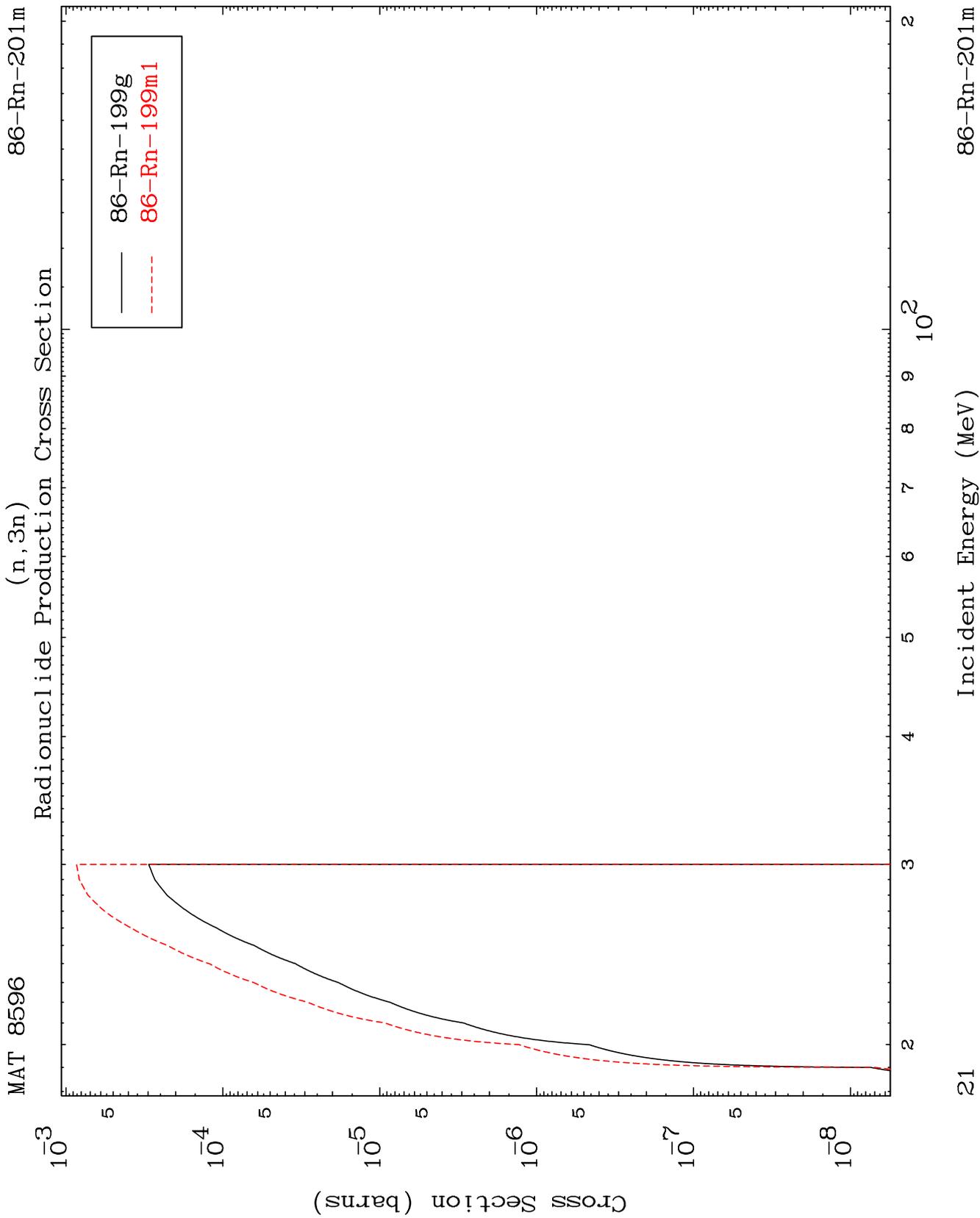
Radionuclide Production Cross Section



20

Incident Energy (MeV)

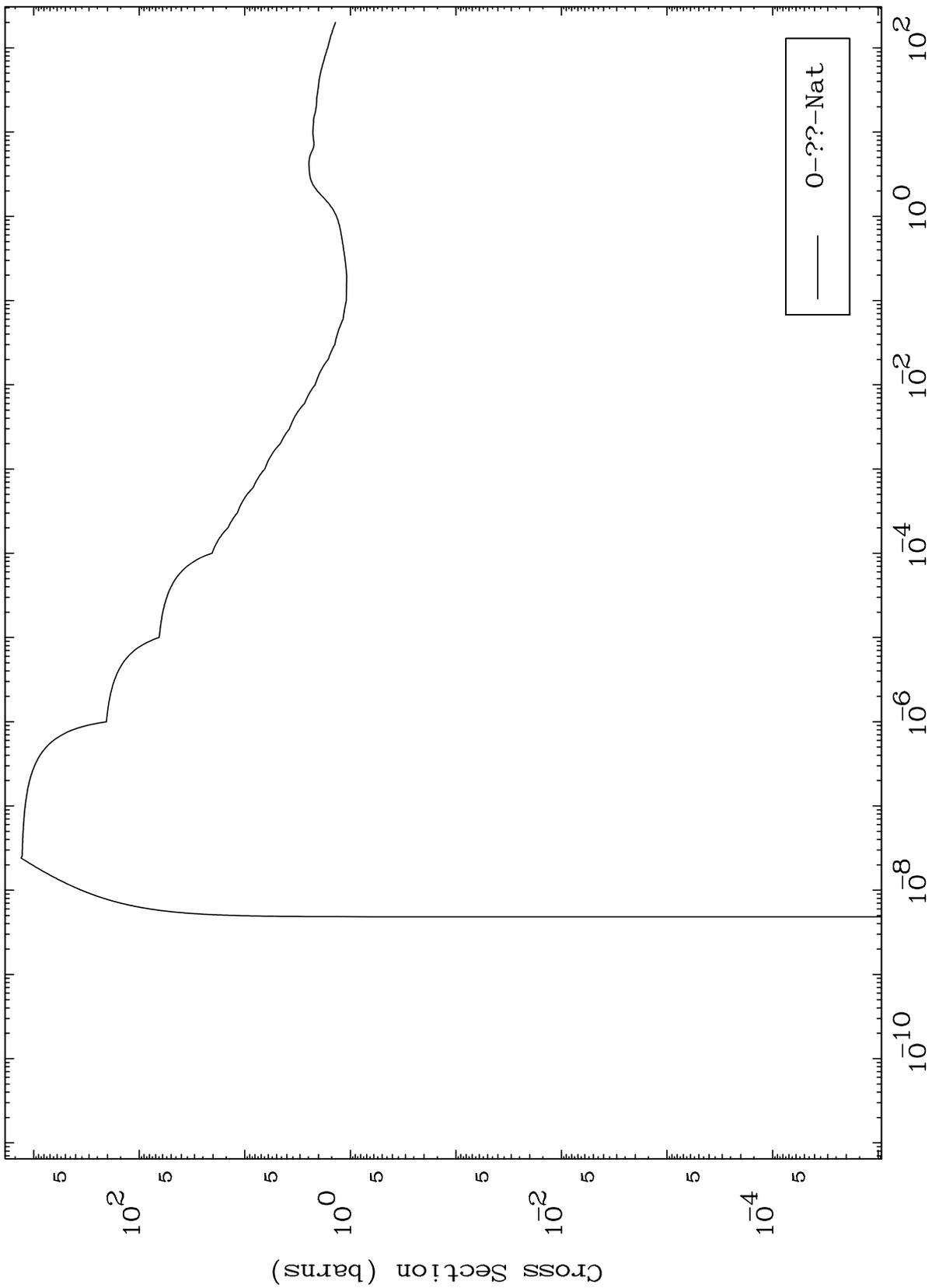
86-Rn-201m



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86-Rn-201m

Fission
Radionuclide Production Cross Section



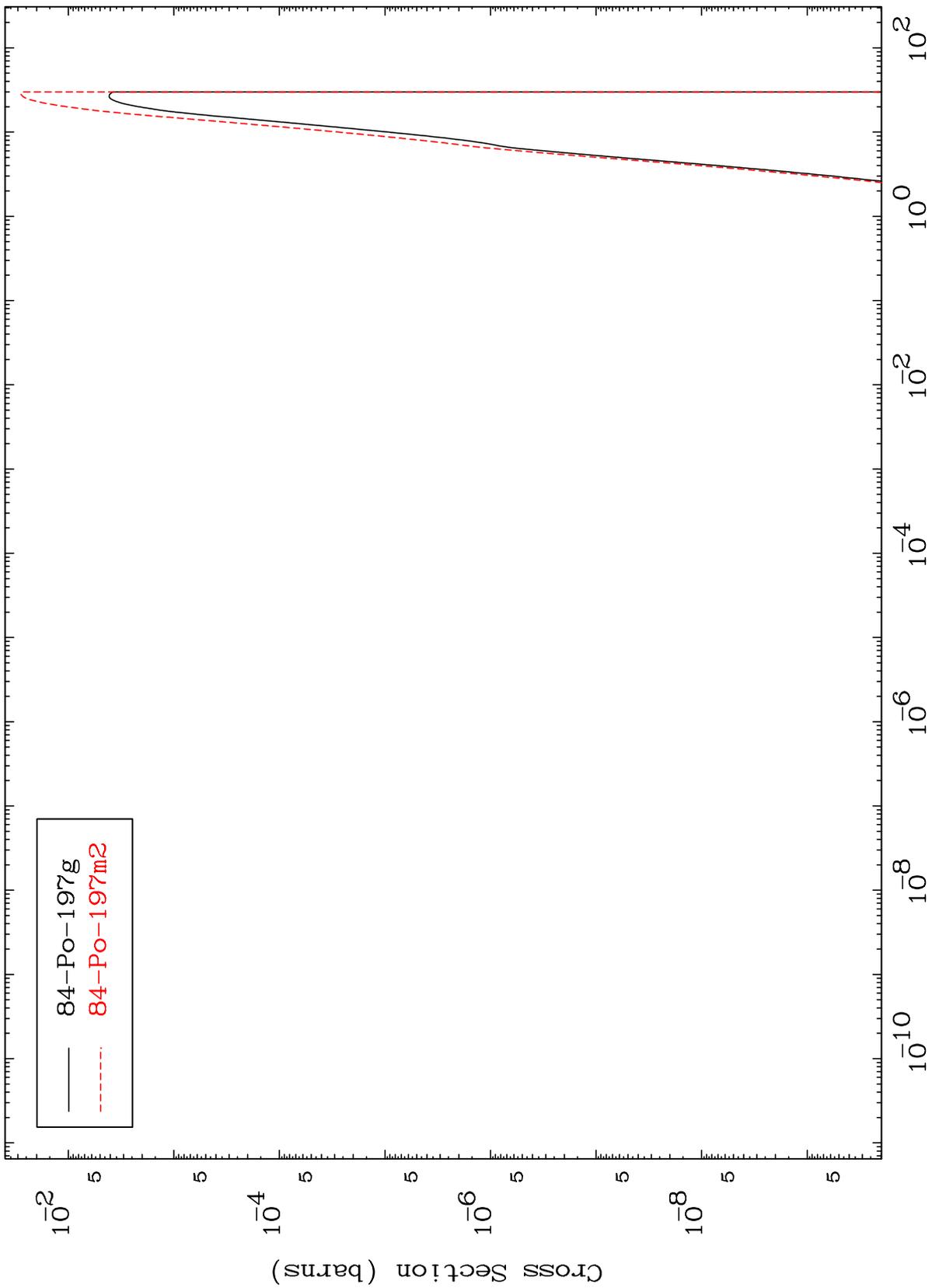
0-??-Nat

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(n, n') α

86-Rn-201m

Radionuclide Production Cross Section



— 84-Po-197g
- - - 84-Po-197m2

Incident Energy (MeV)

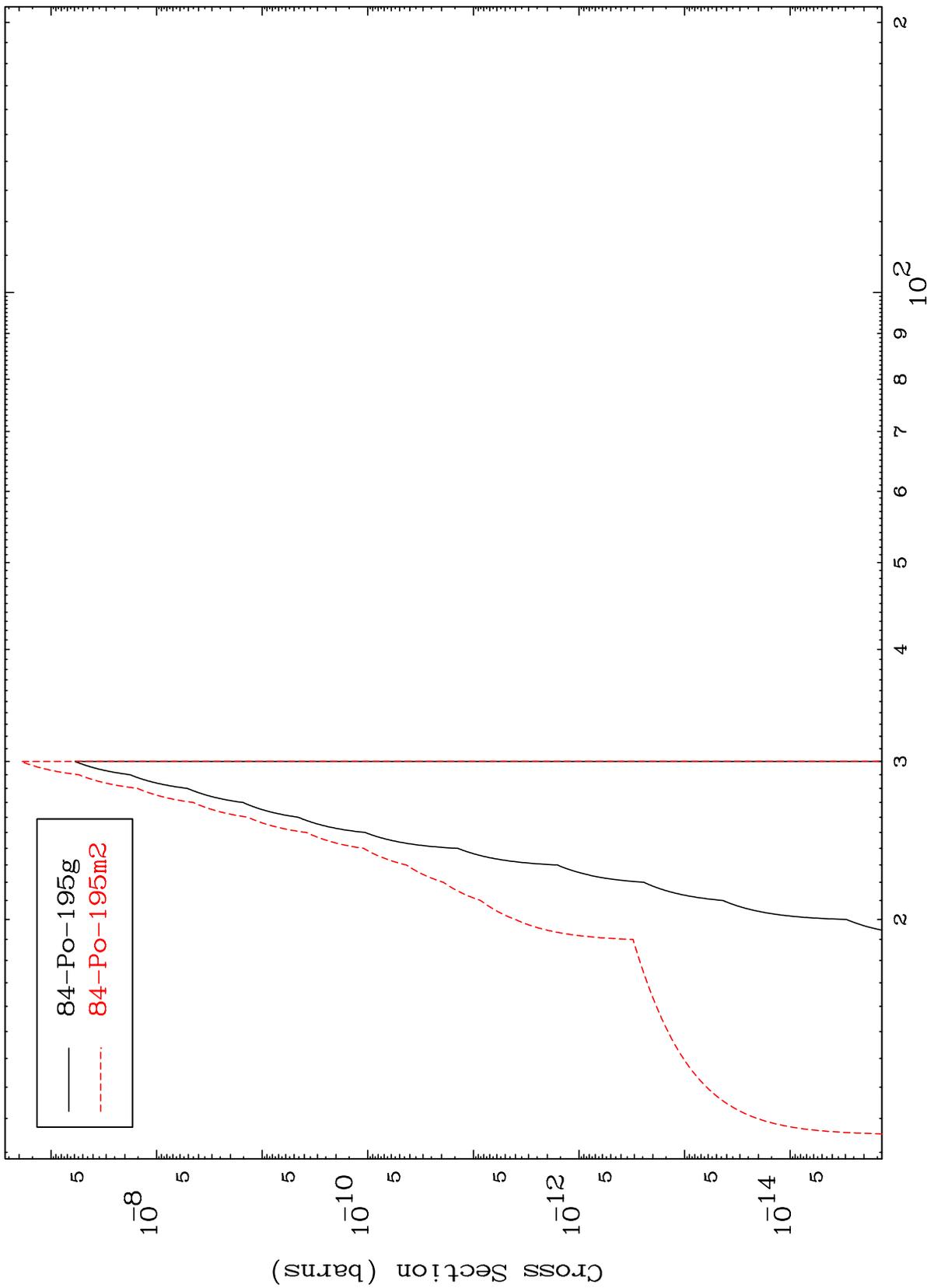
86-Rn-201m

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(n,3n) α

86-Rn-201m

Radionuclide Production Cross Section



24

Incident Energy (MeV)

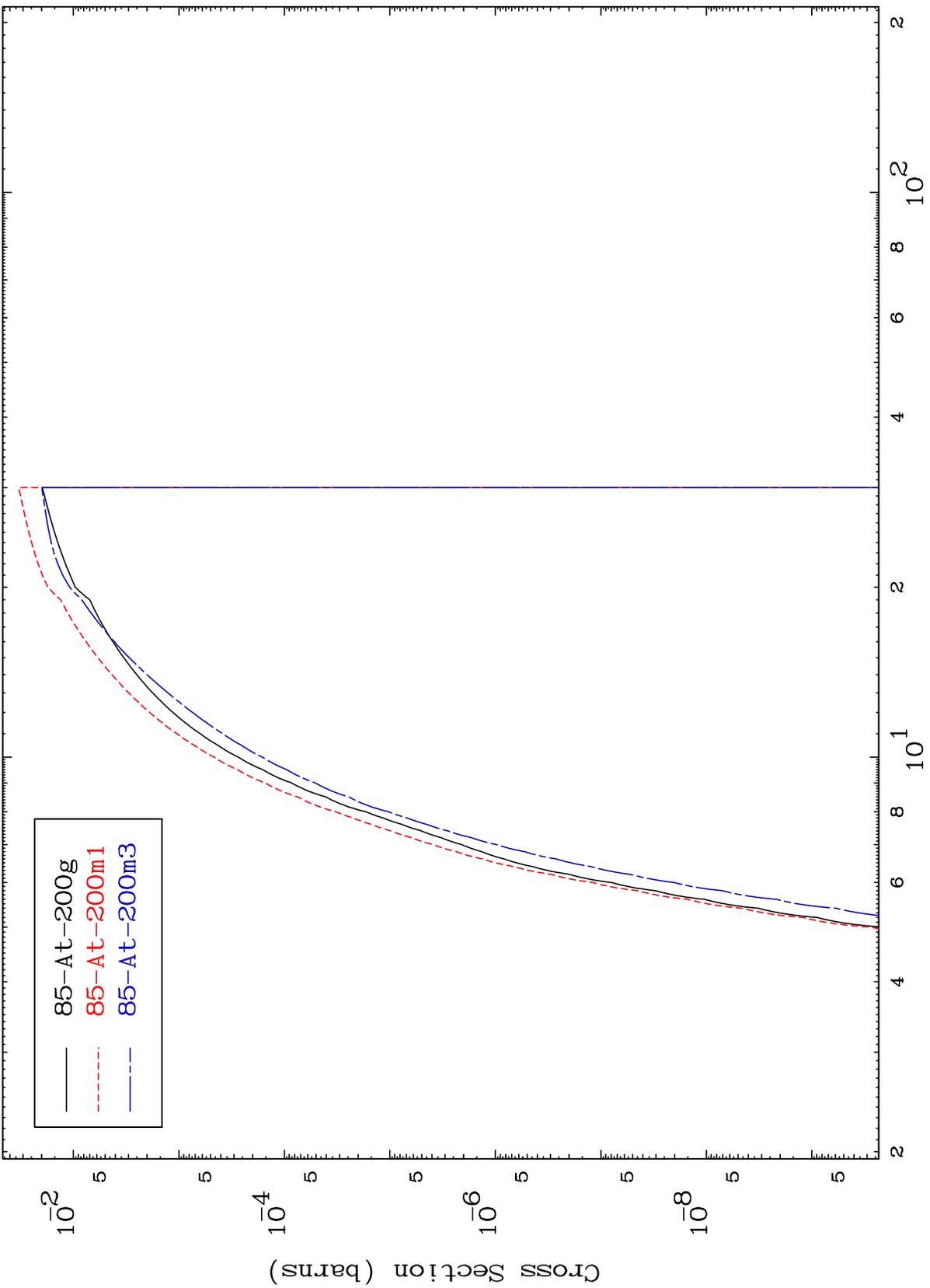
86-Rn-201m

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(n,n') p

86-Rn-201m

Radionuclide Production Cross Section



25

Incident Energy (MeV)

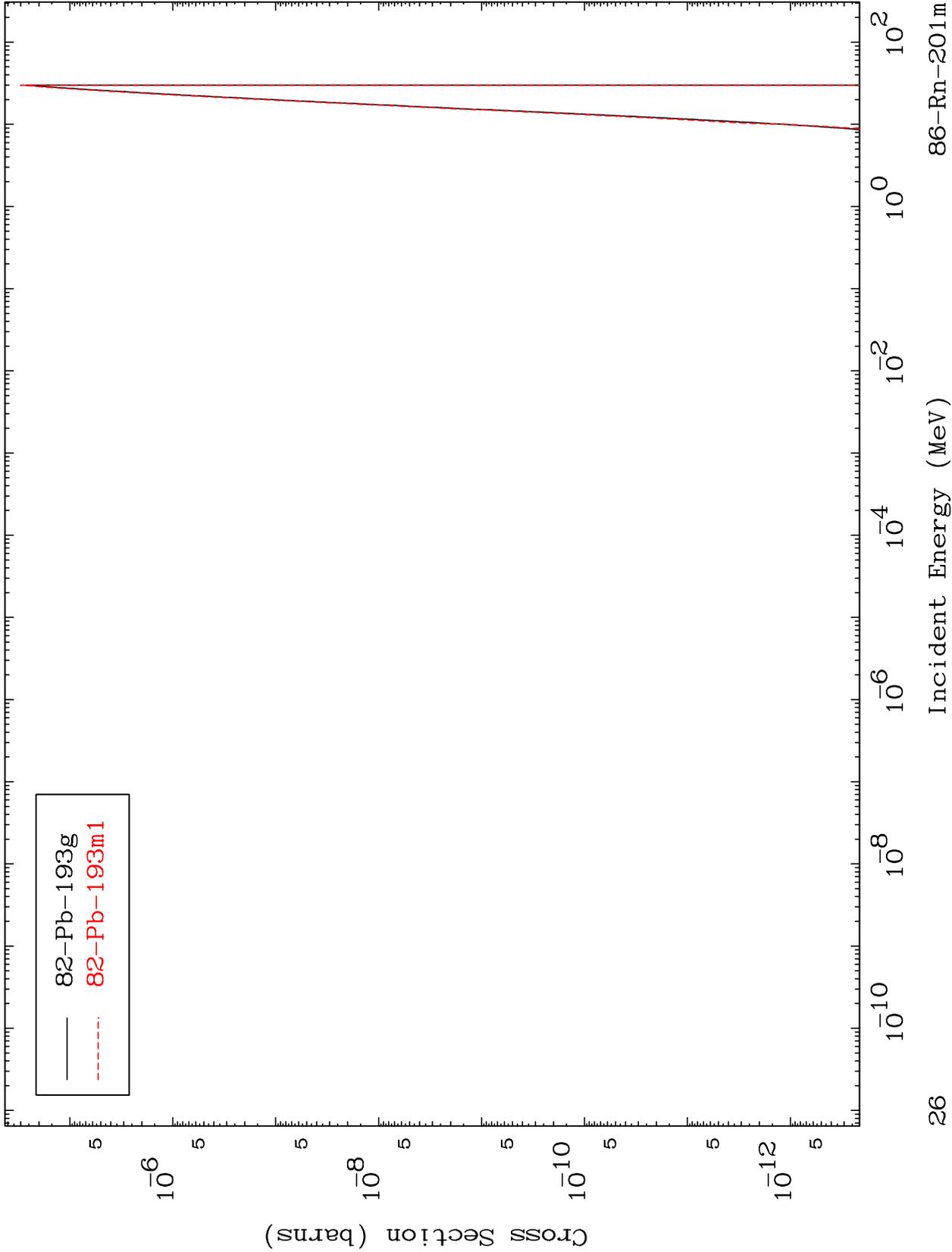
86-Rn-201m

MAT 8596

(n,n') 2 α

86-Rn-201m

Radionuclide Production Cross Section

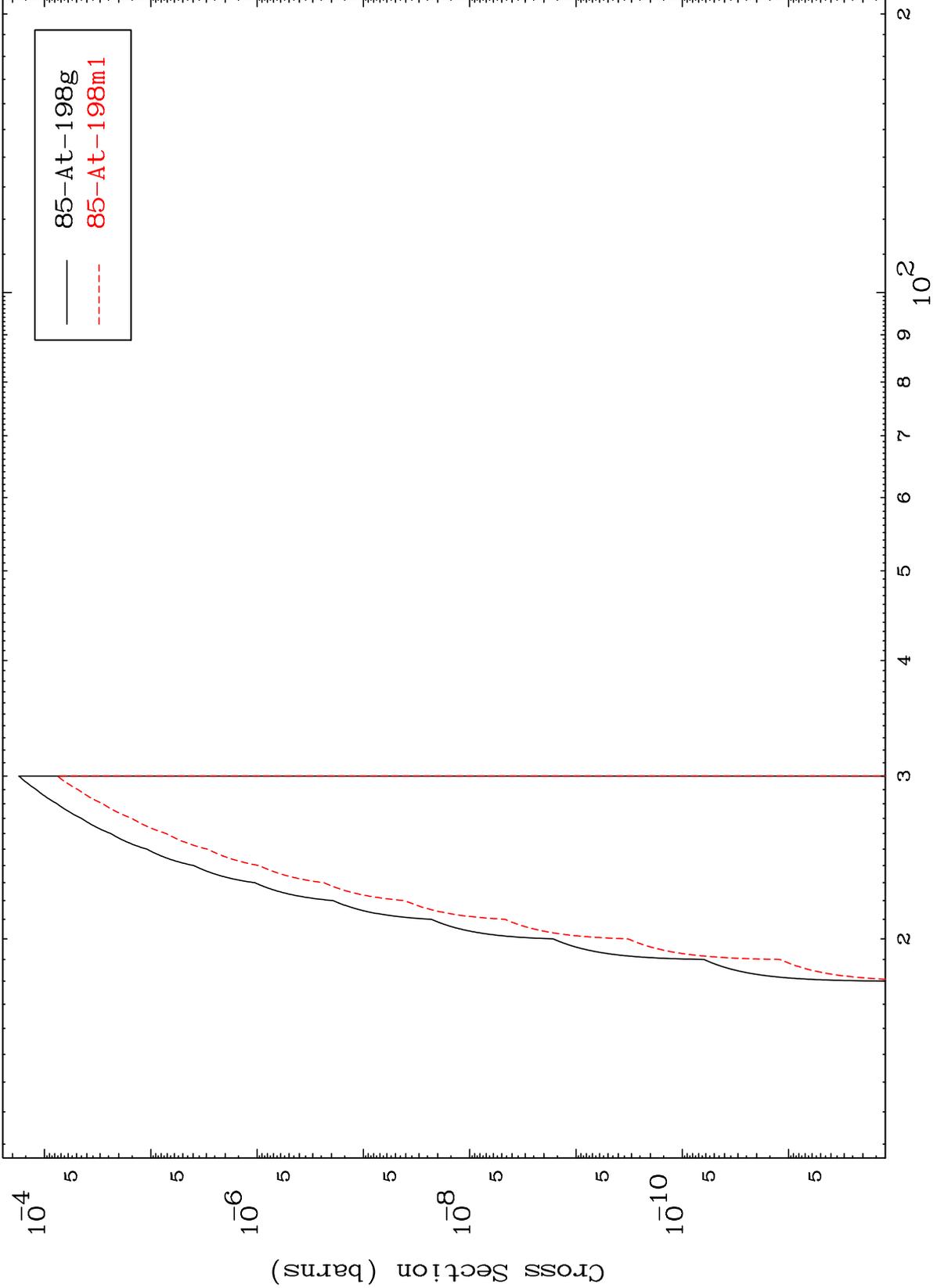


MAT 8596

(n,n') t

86-Rn-201m

Radionuclide Production Cross Section



27

Incident Energy (MeV)

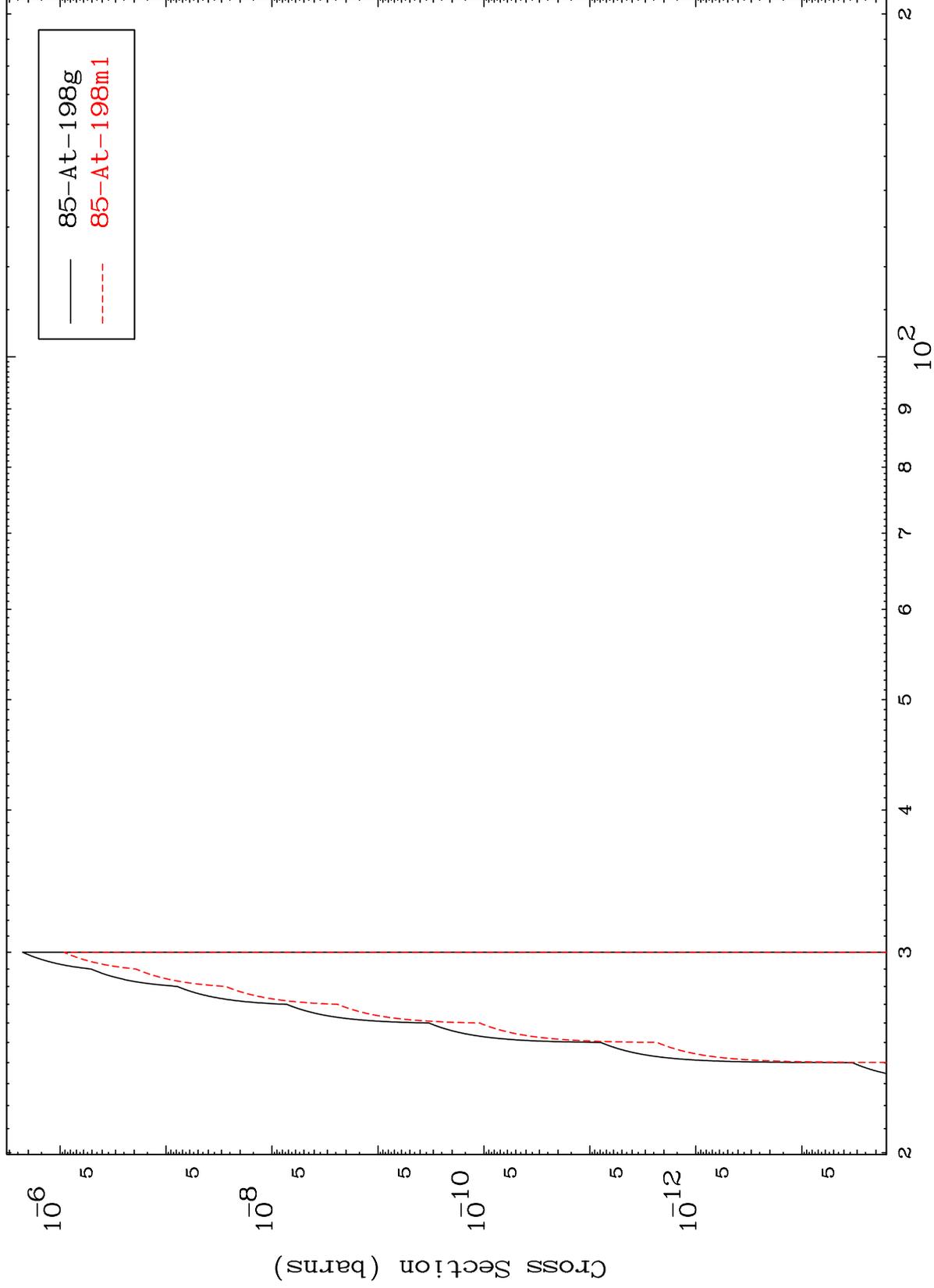
86-Rn-201m

MAT 8596

(n,3n) p

86-Rn-201m

Radionuclide Production Cross Section



28

Incident Energy (MeV)

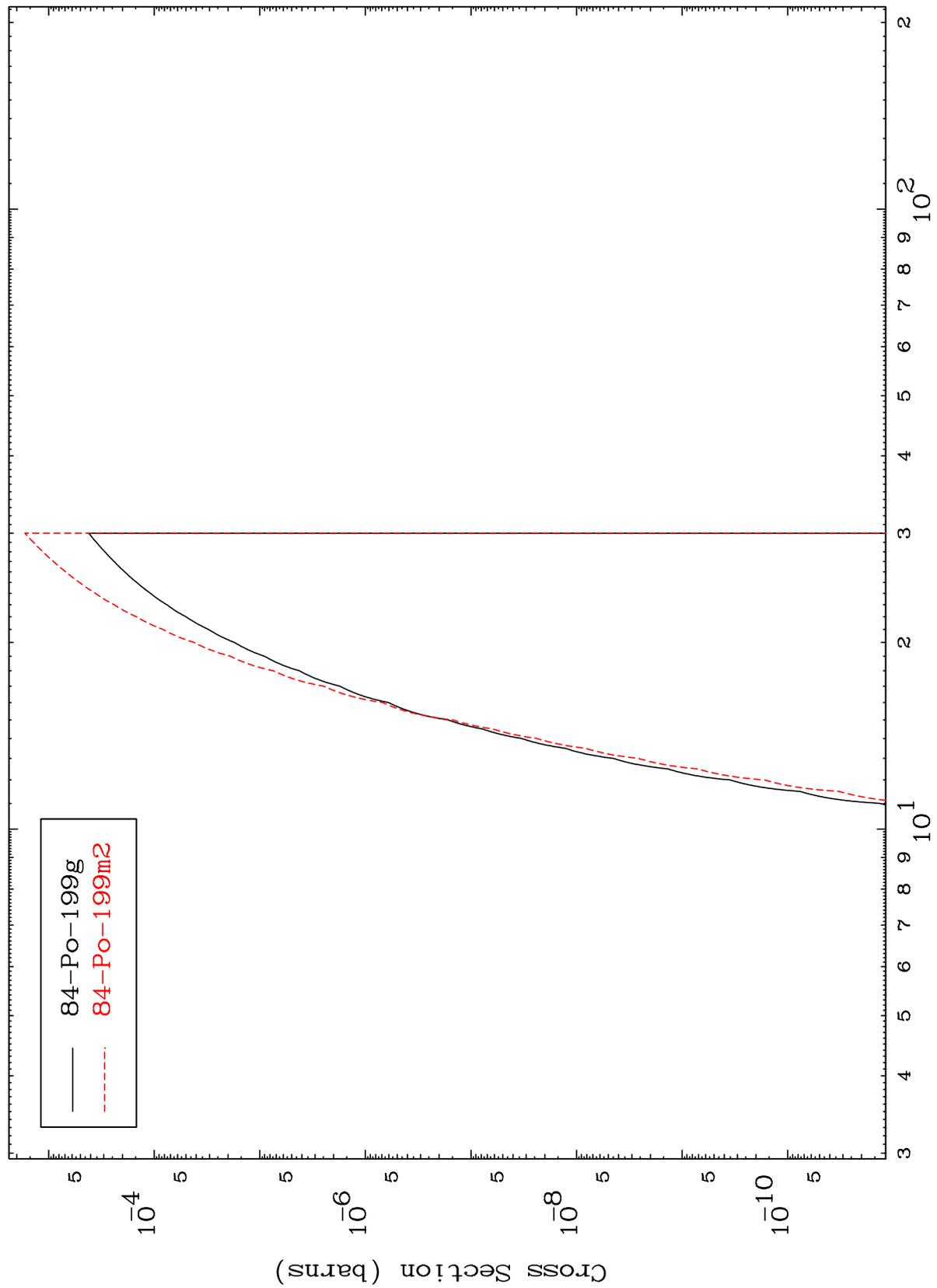
86-Rn-201m

MAT 8596

(n,2n) p

86-Rn-201m

Radionuclide Production Cross Section



29

Incident Energy (MeV)

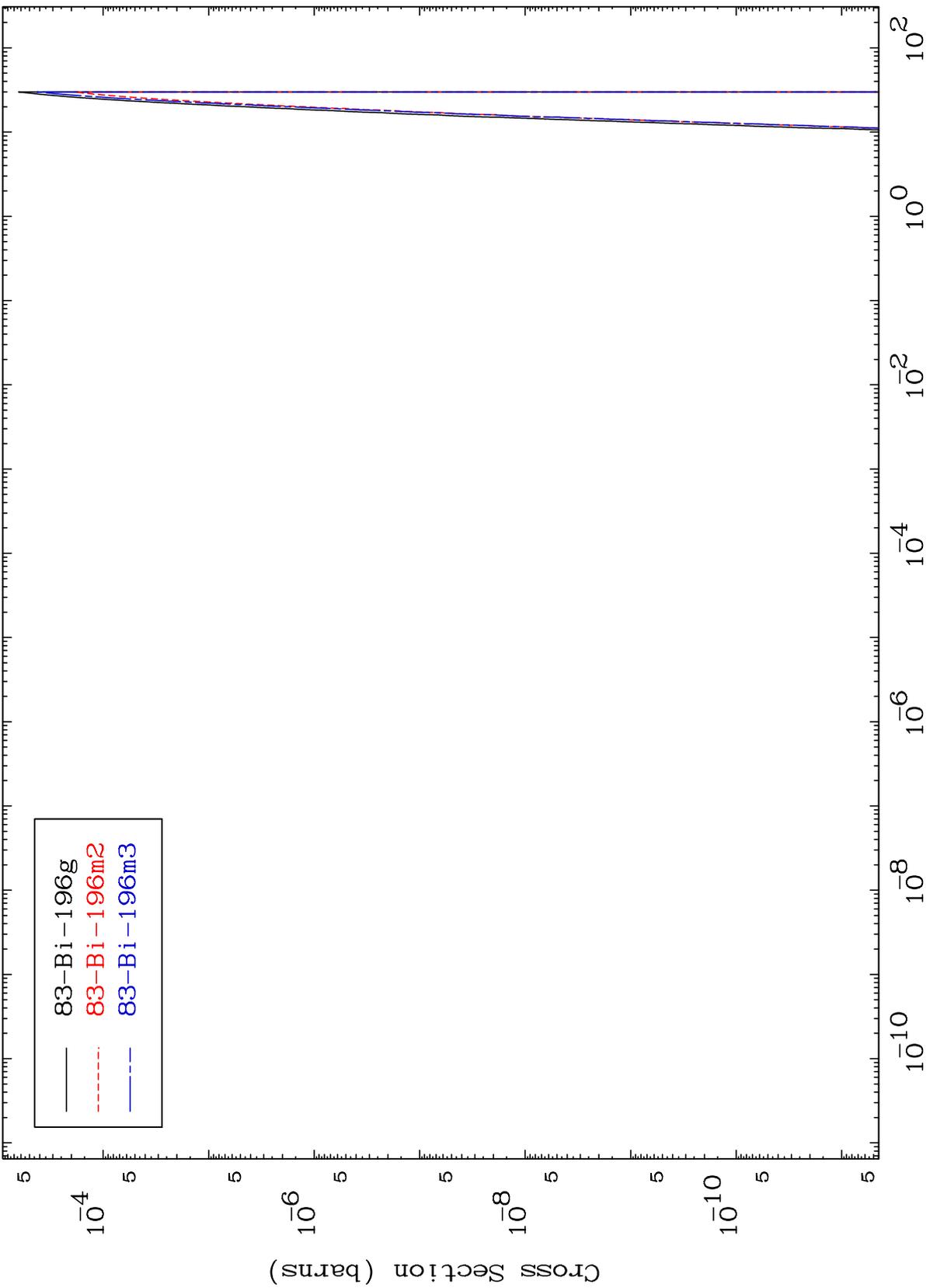
86-Rn-201m

MAT 8596

(n,n') p α

86-Rn-201m

Radionuclide Production Cross Section



30

Incident Energy (MeV)

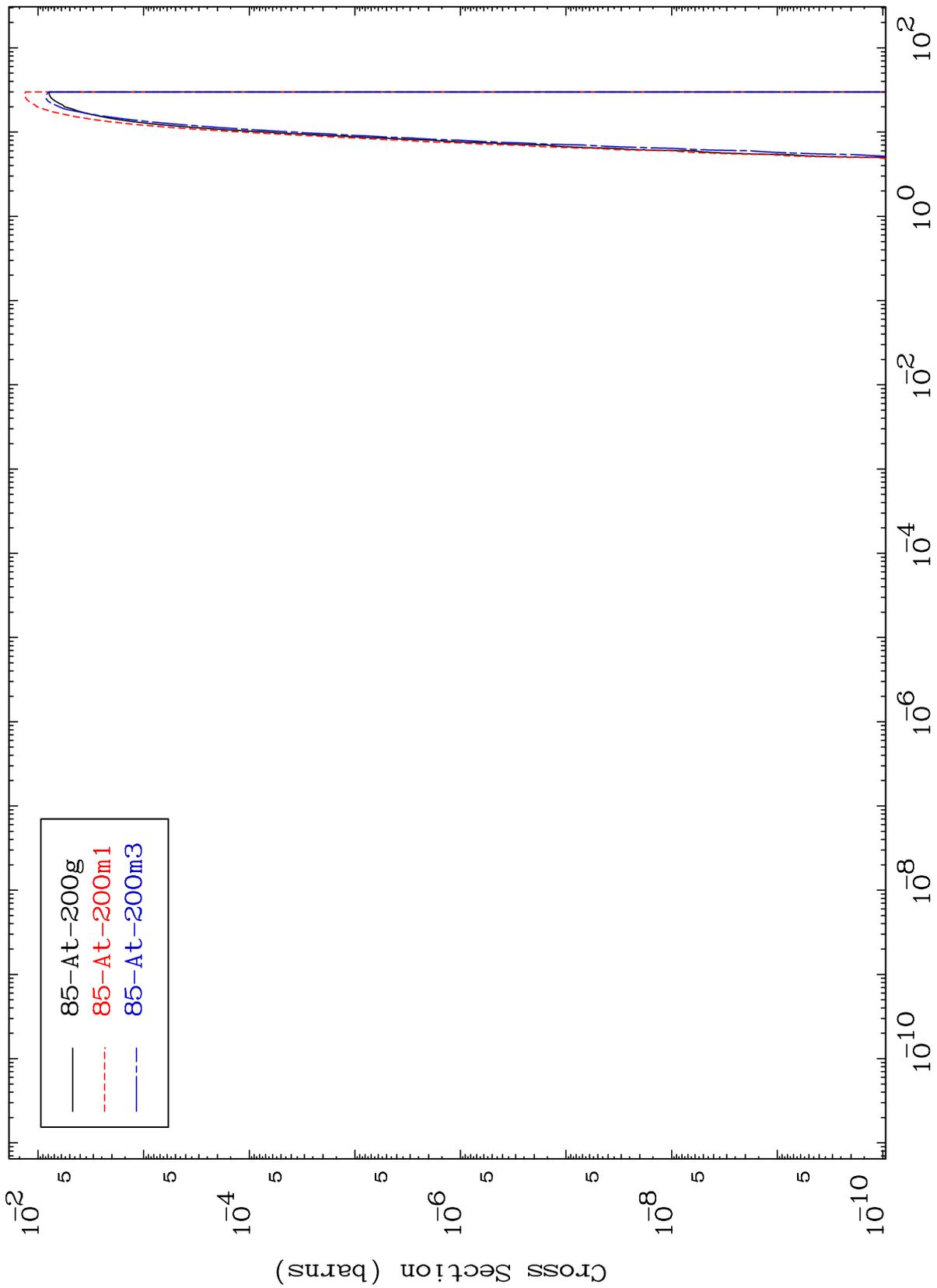
86-Rn-201m

MAT 8596

(n,d)

86-Rn-201m

Radionuclide Production Cross Section



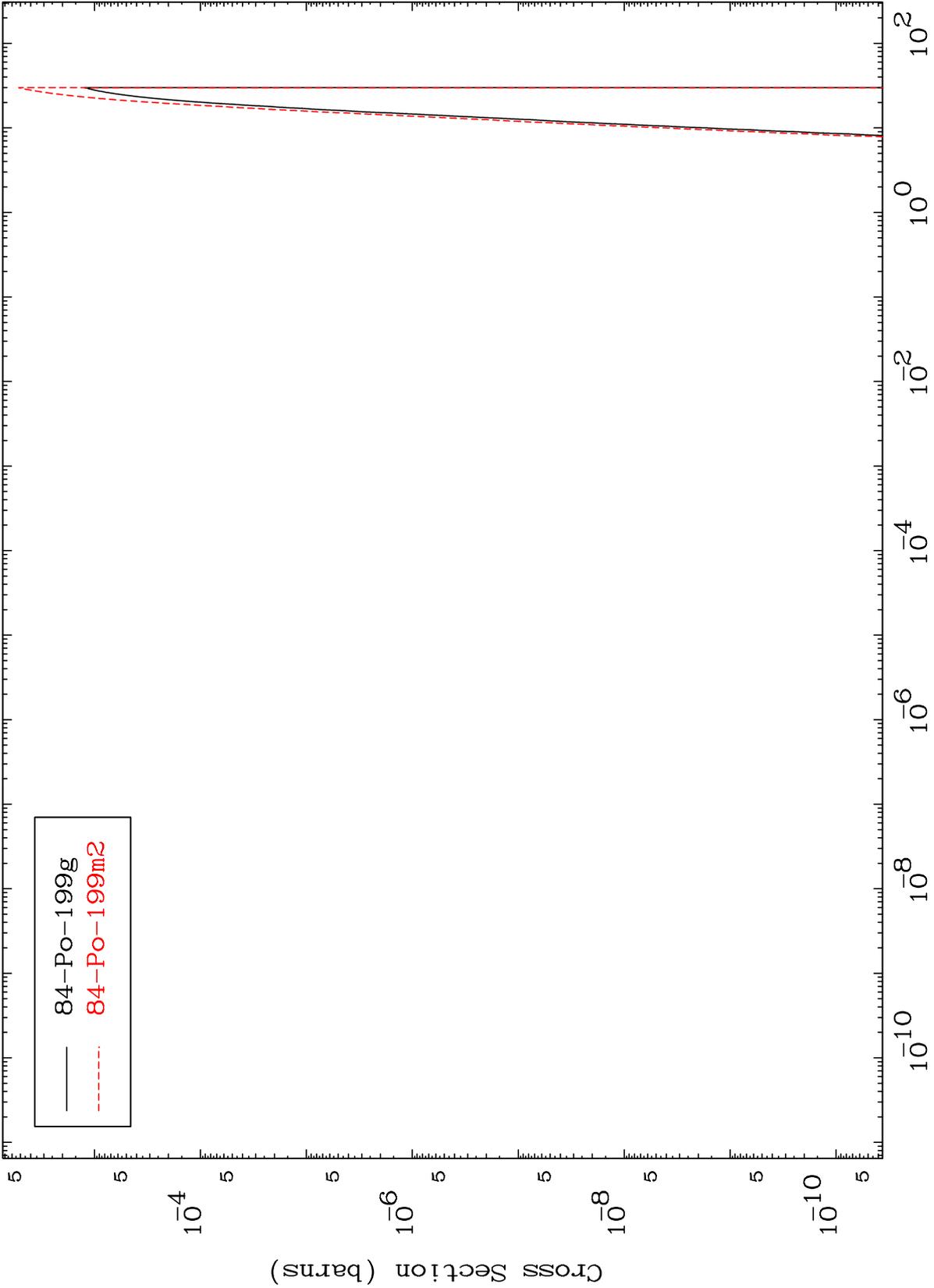
86-Rn-201m

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(n,He-3)

86-Rn-201m

Radionuclide Production Cross Section



32

Incident Energy (MeV)

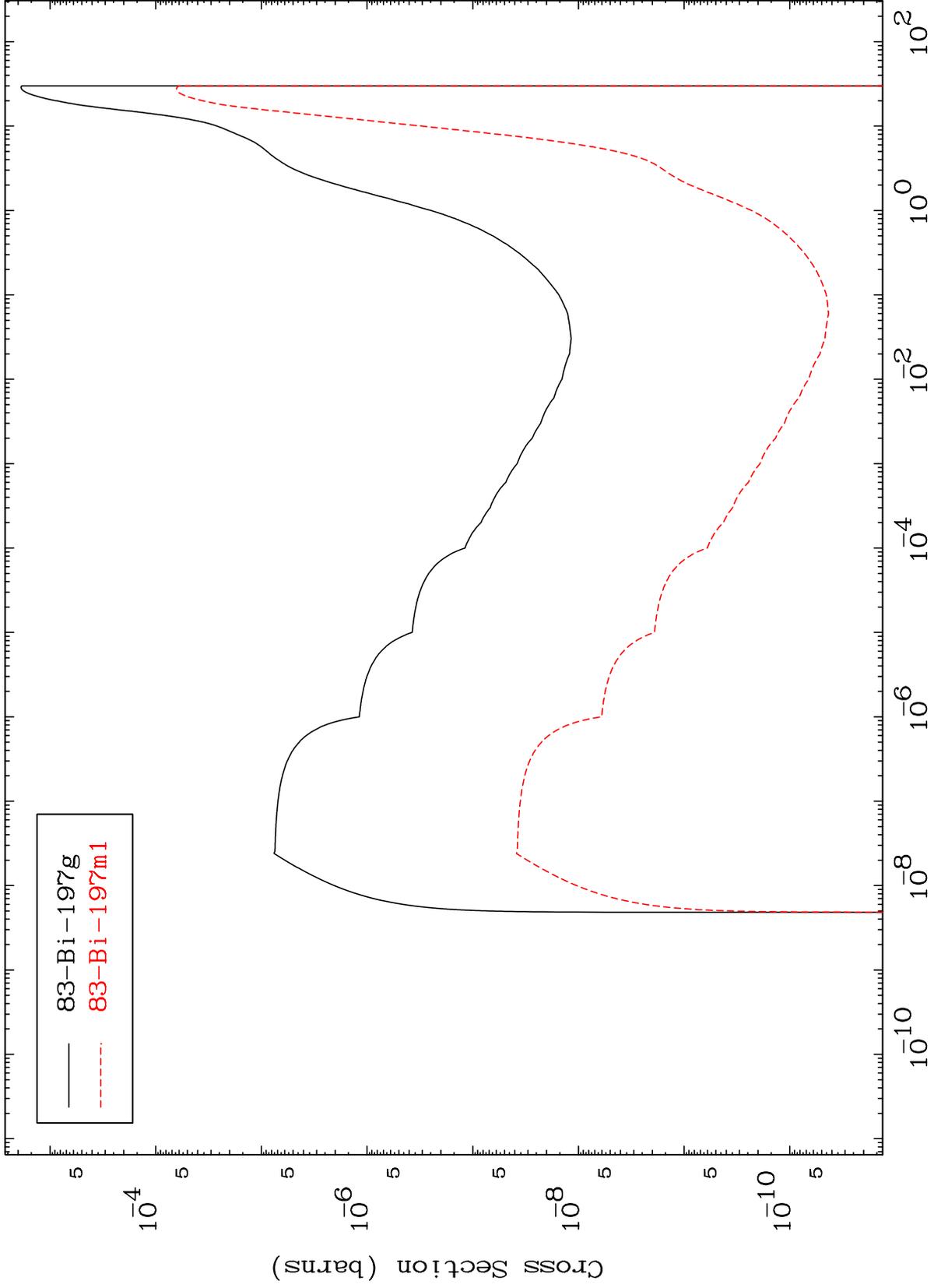
86-Rn-201m

MAT 8596

(n,p) α

86-Rn-201m

Radionuclide Production Cross Section

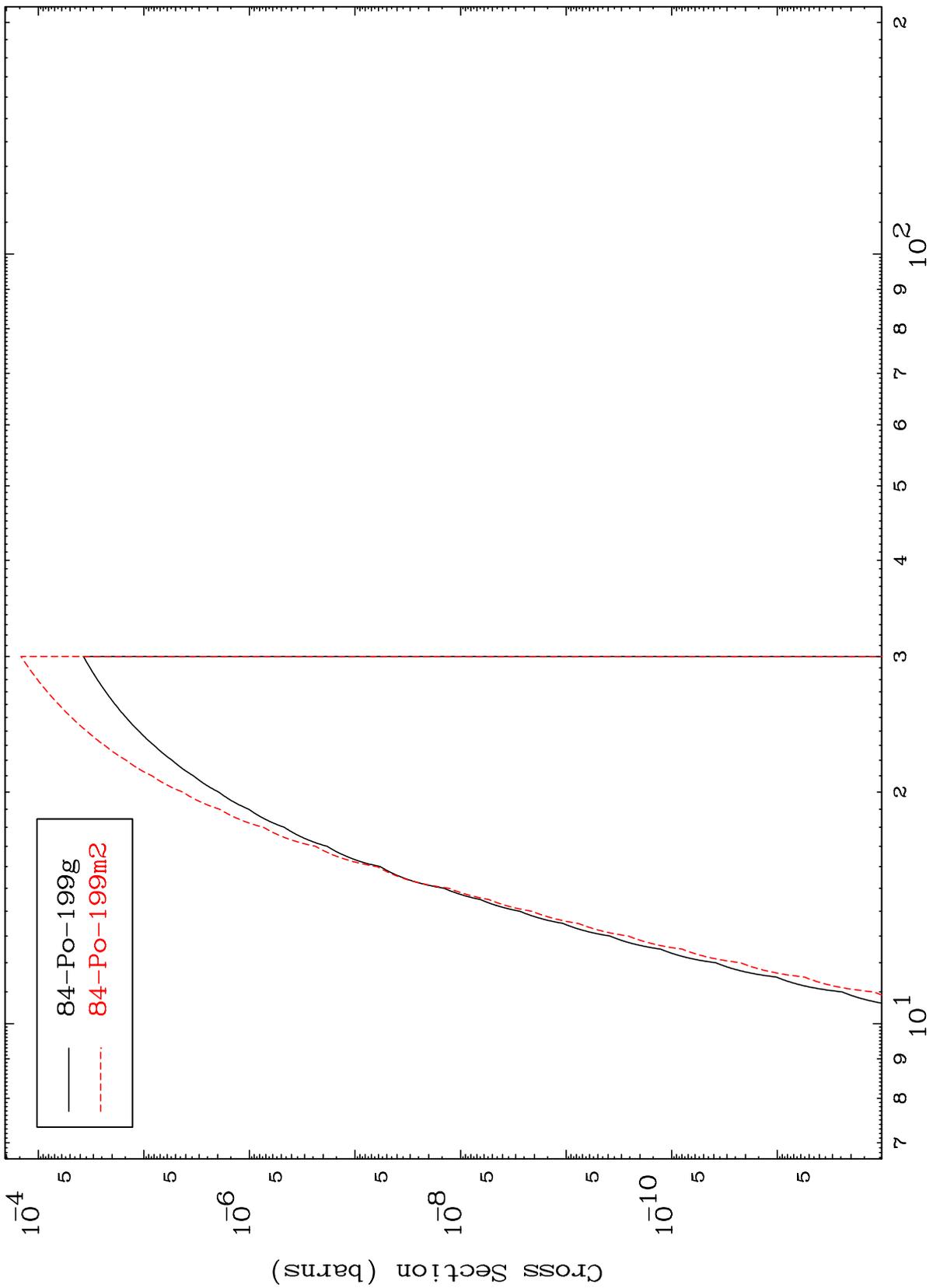


MAT 8596

(n,p) d

86-Rn-201m

Radionuclide Production Cross Section



34

Incident Energy (MeV)

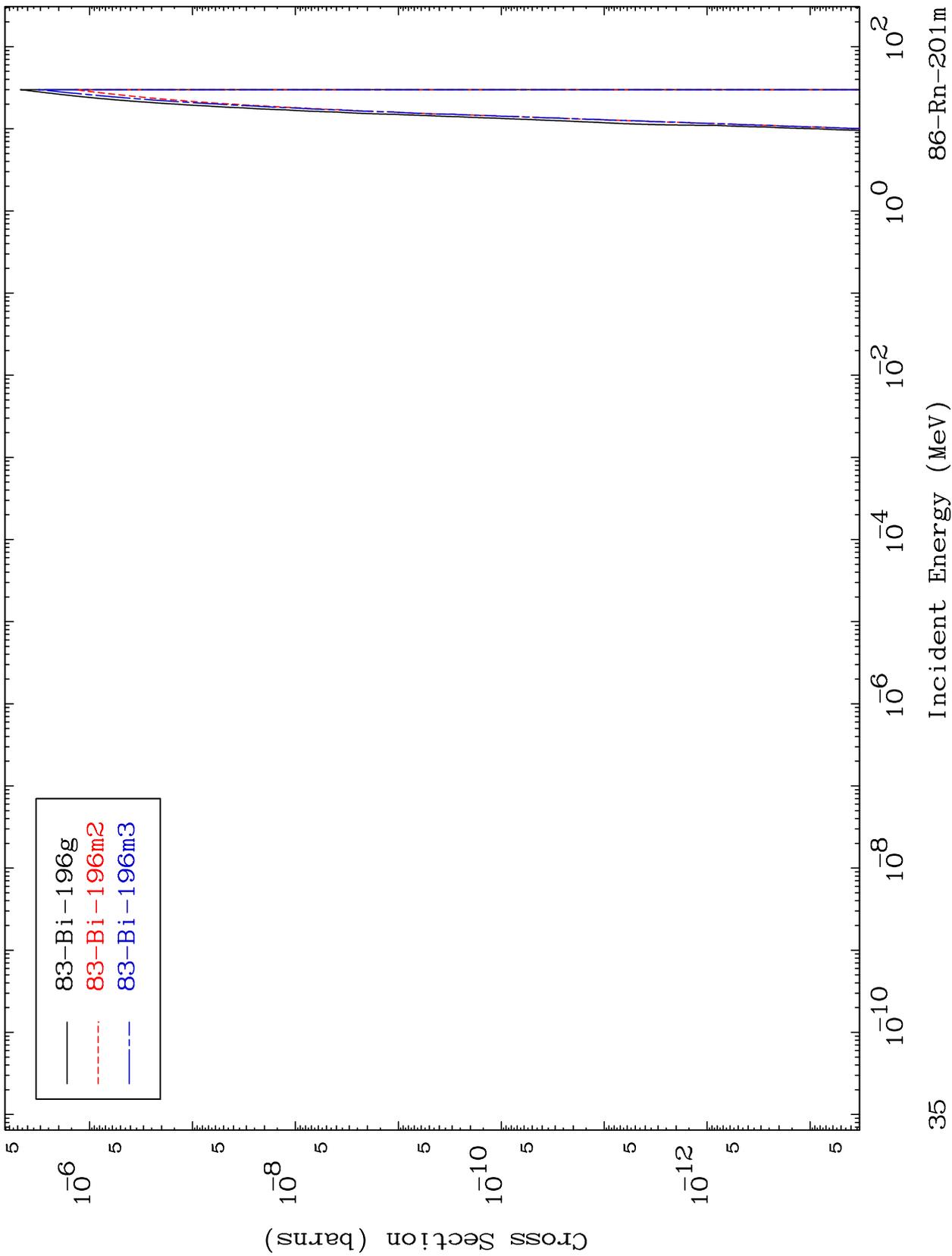
86-Rn-201m

MAT 8596

(n,d) α

86-Rn-201m

Radionuclide Production Cross Section



35

Incident Energy (MeV)

86-Rn-201m