

Program EVALPLOT  
(Version 2021-1)

by

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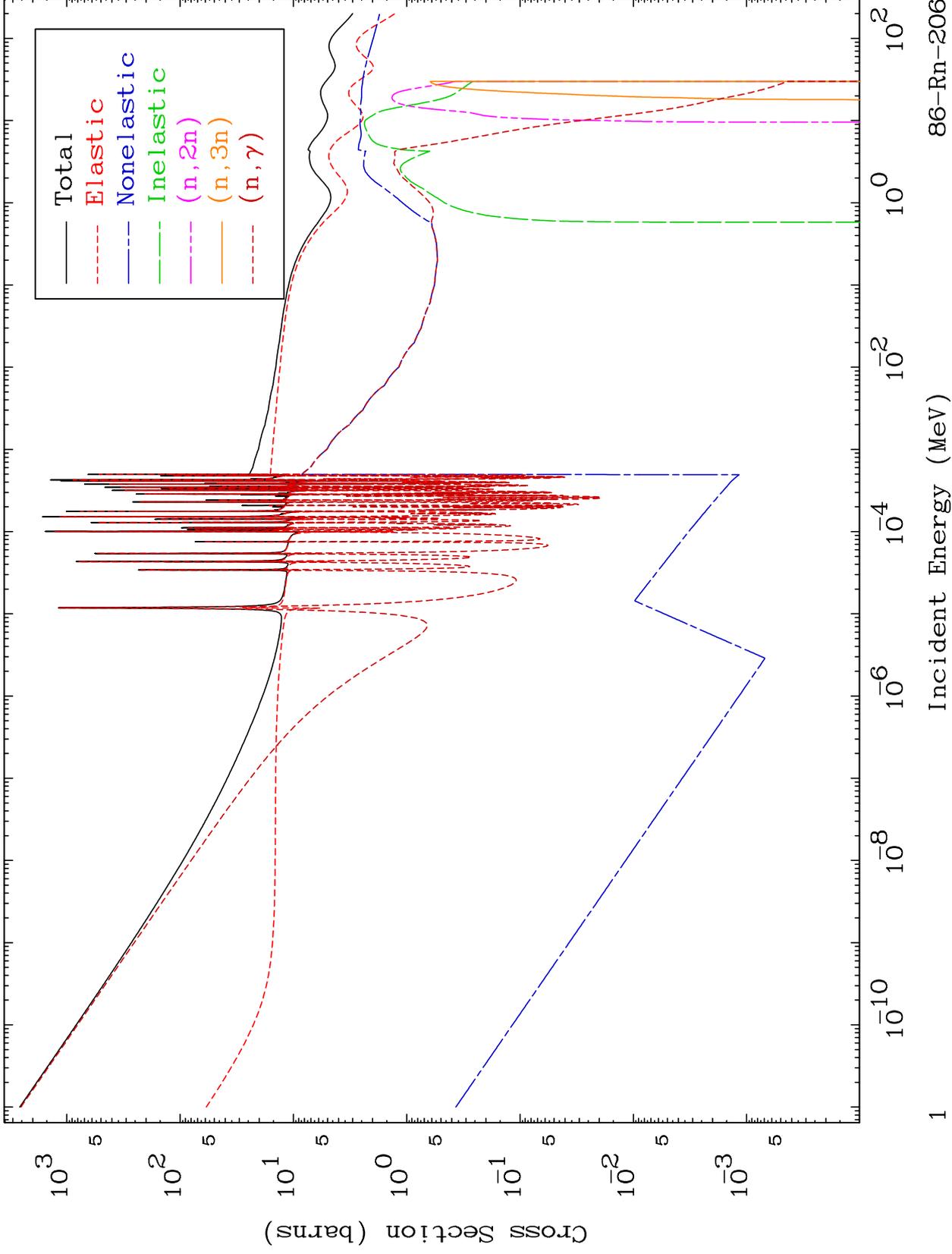
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Press Mouse Button to Start

MAT 8610

Neutron Major  
293 Kelvin Cross Sections

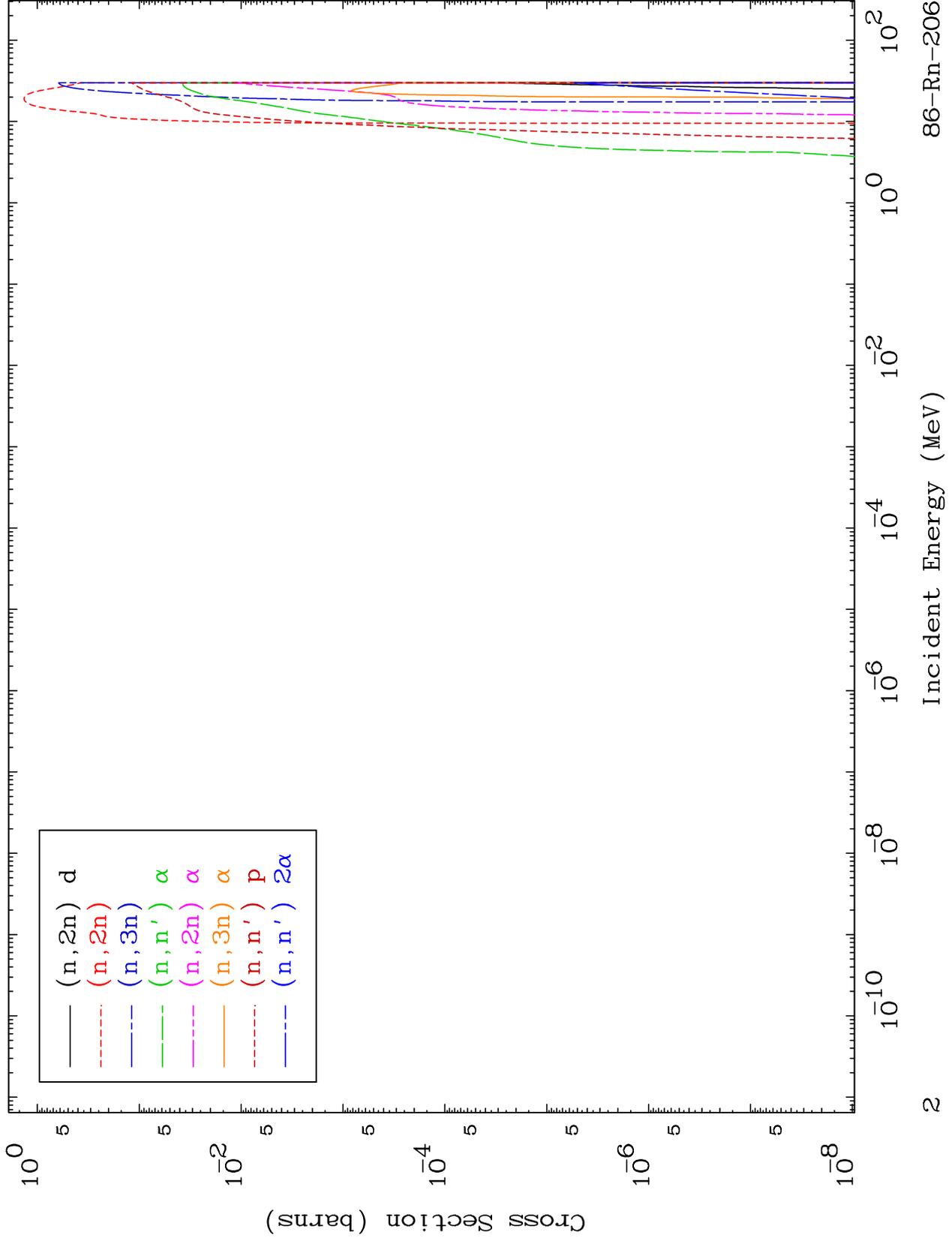
86-Rn-206



MAT 8610

Neutron Absorption  
293 Kelvin Cross Sections

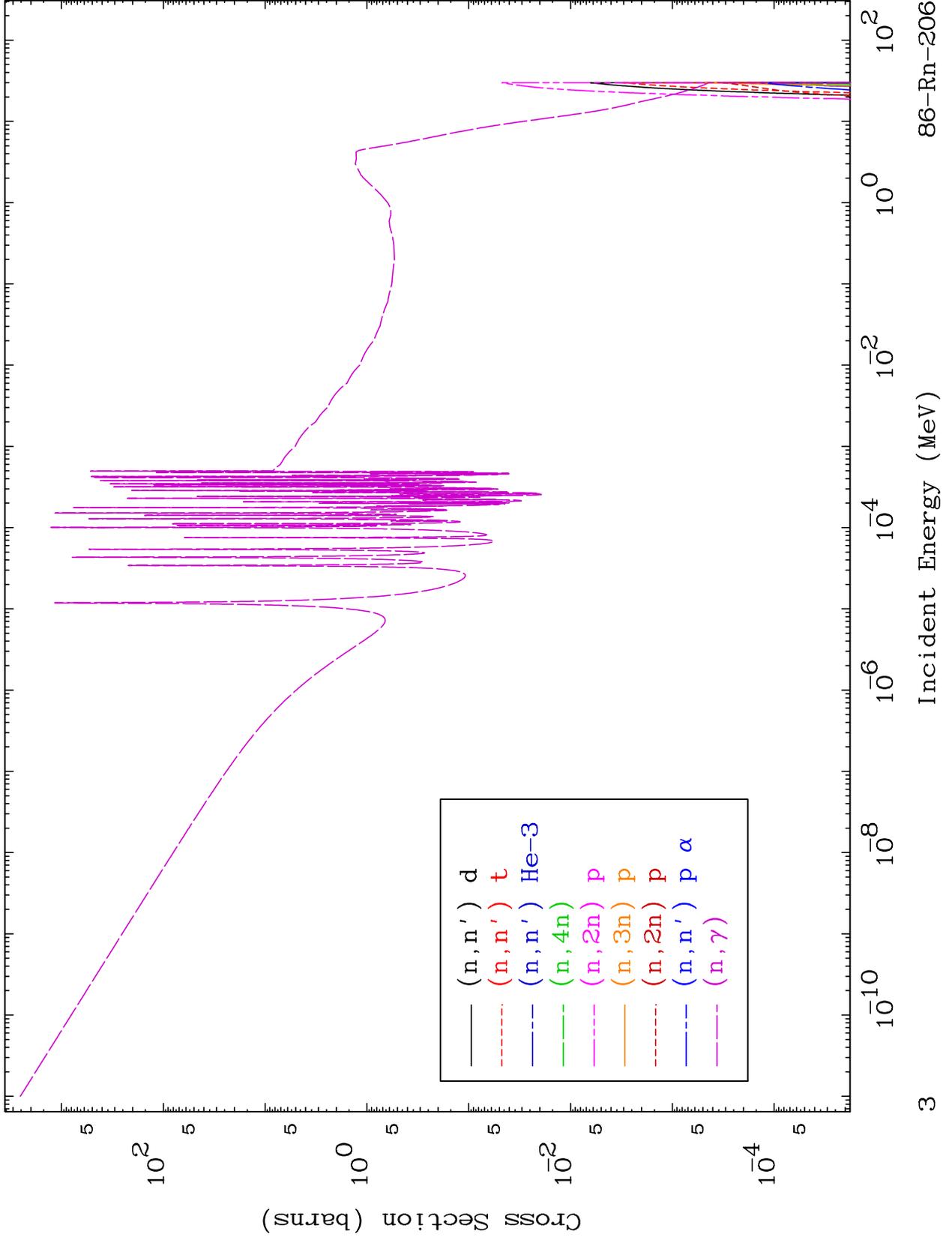
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MAT 8610

Neutron Absorption  
293 Kelvin Cross Sections

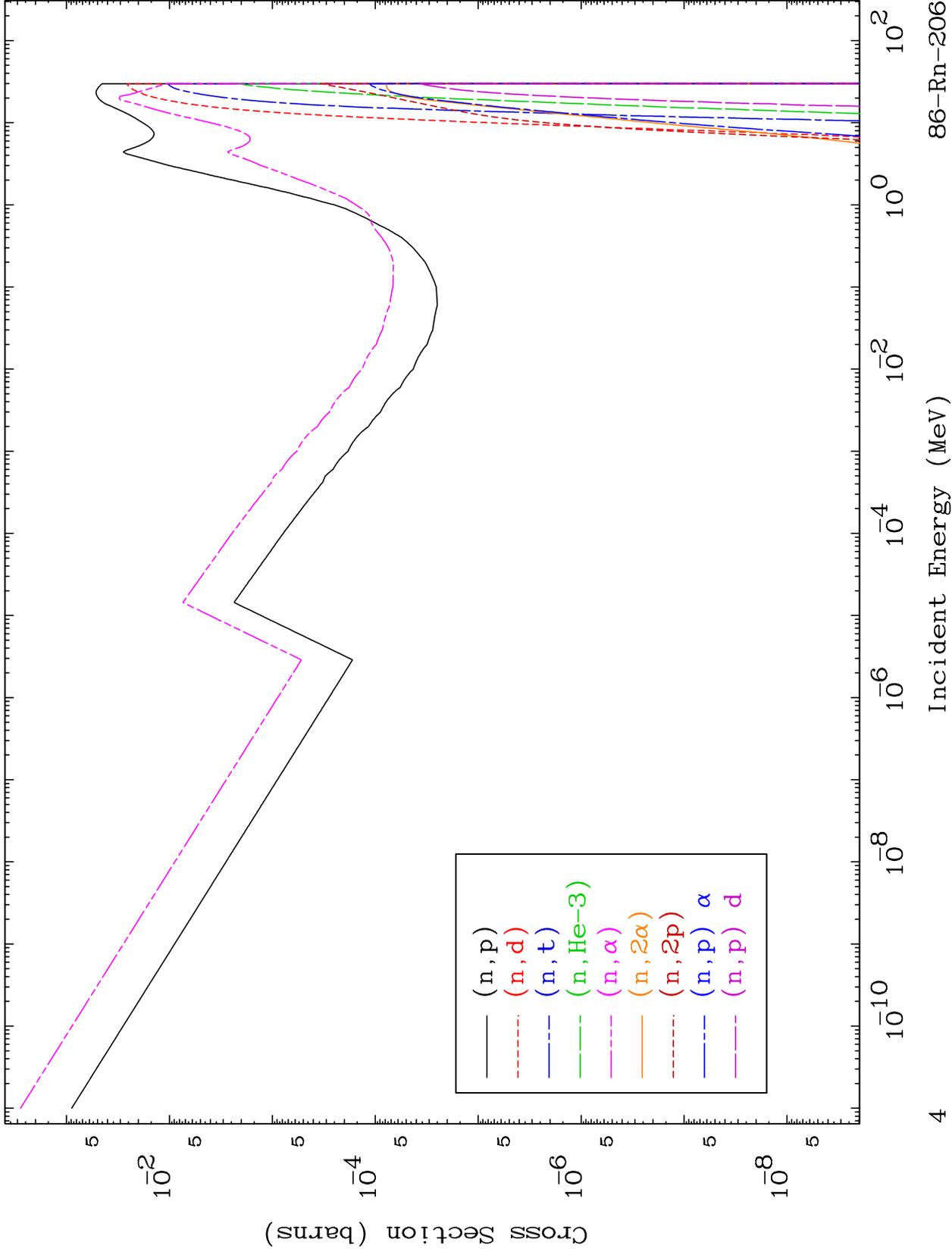
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MAT 8610

Neutron Absorption  
293 Kelvin Cross Sections

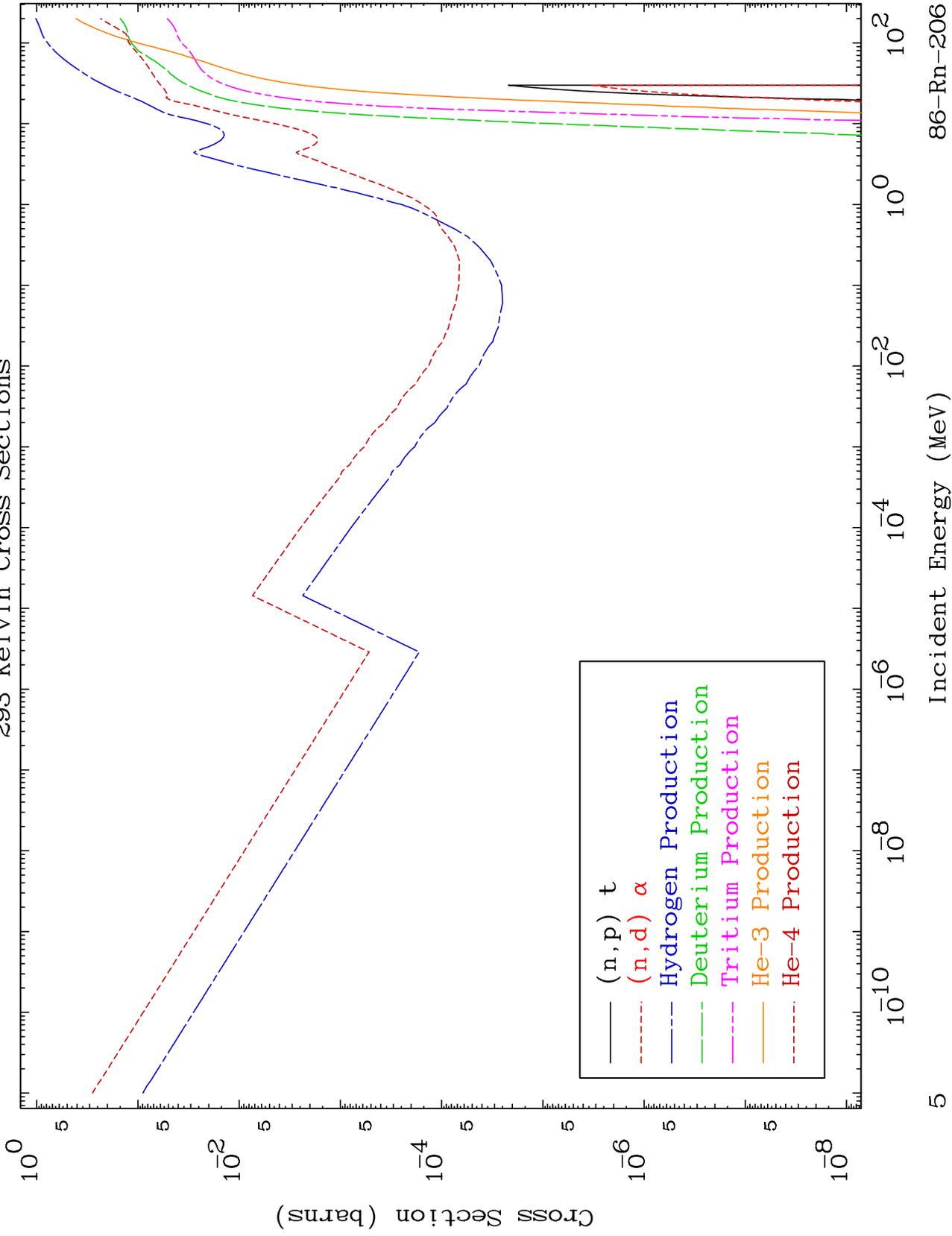
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MAT 8610

Neutron Absorption  
293 Kelvin Cross Sections

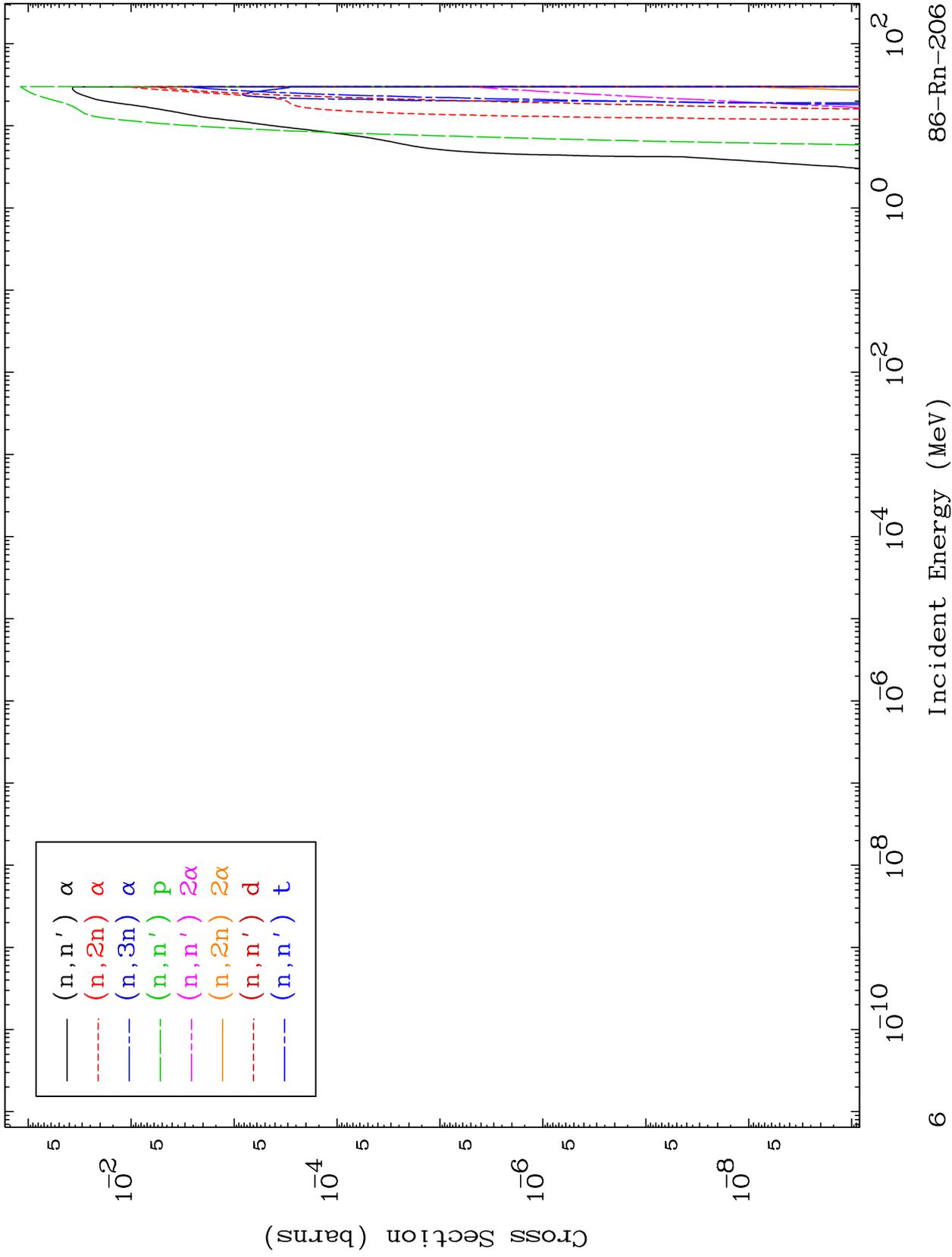
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MAT 8610

Charged Particle  
293 Kelvin Cross Sections

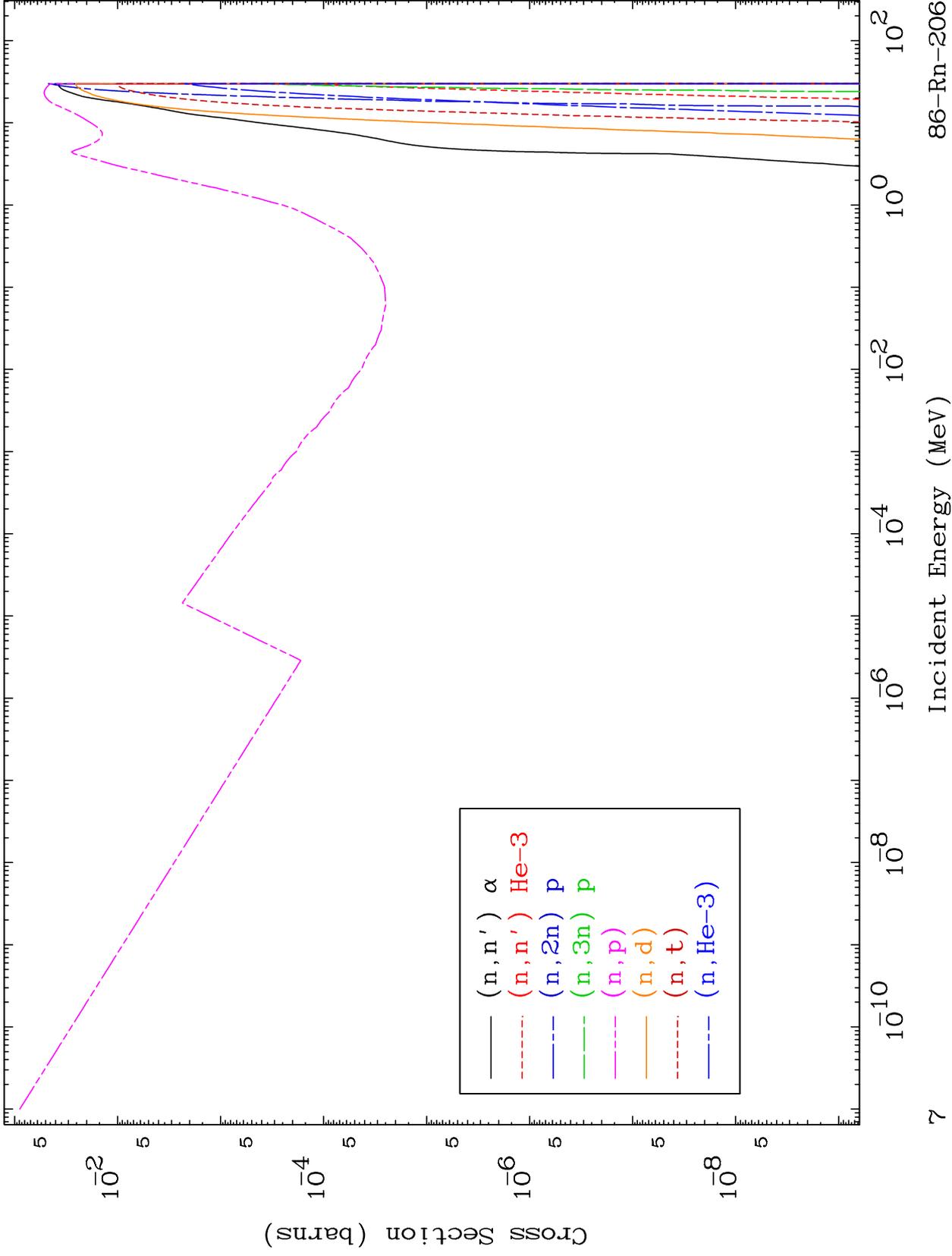
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MAT 8610

Charged Particle  
293 Kelvin Cross Sections

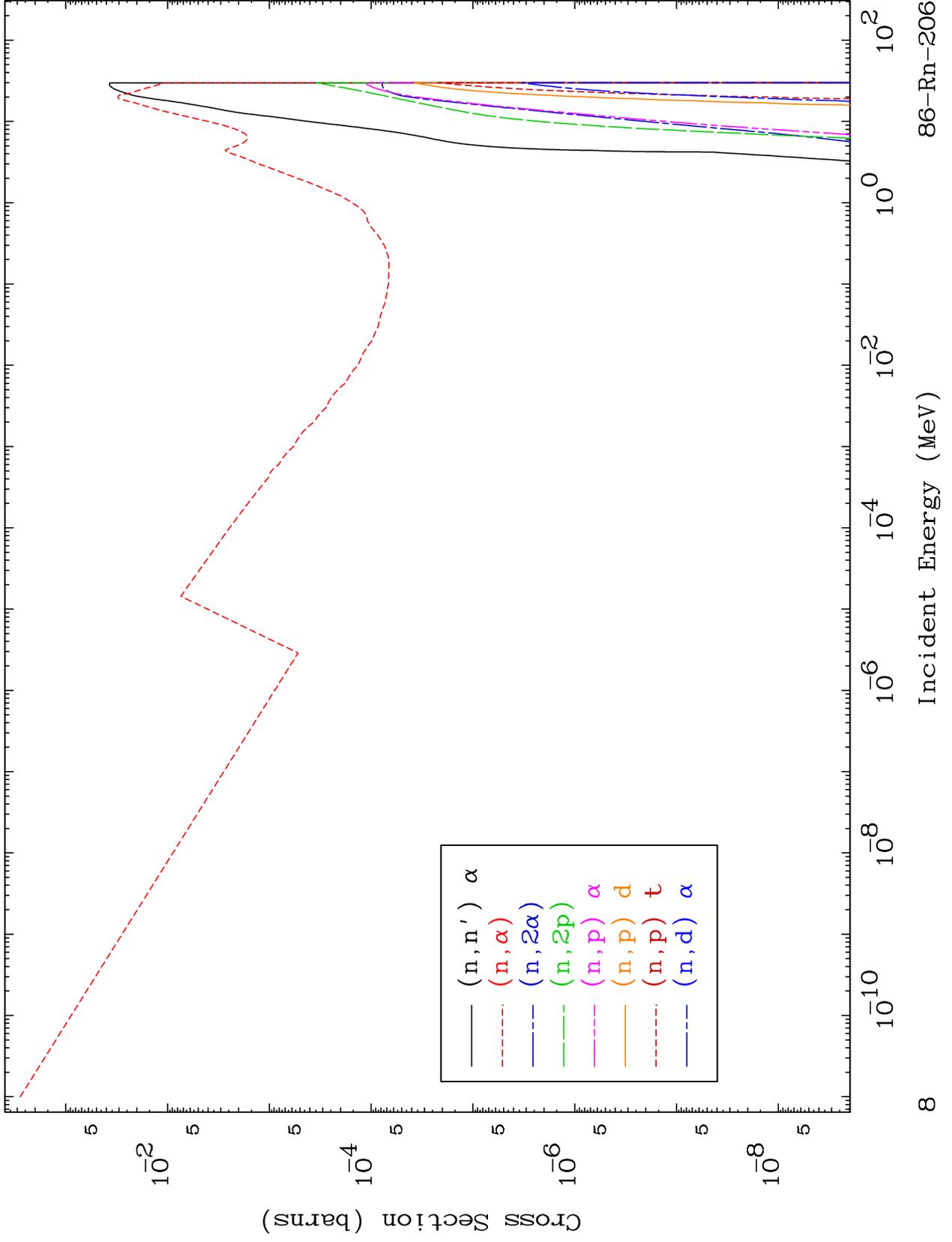
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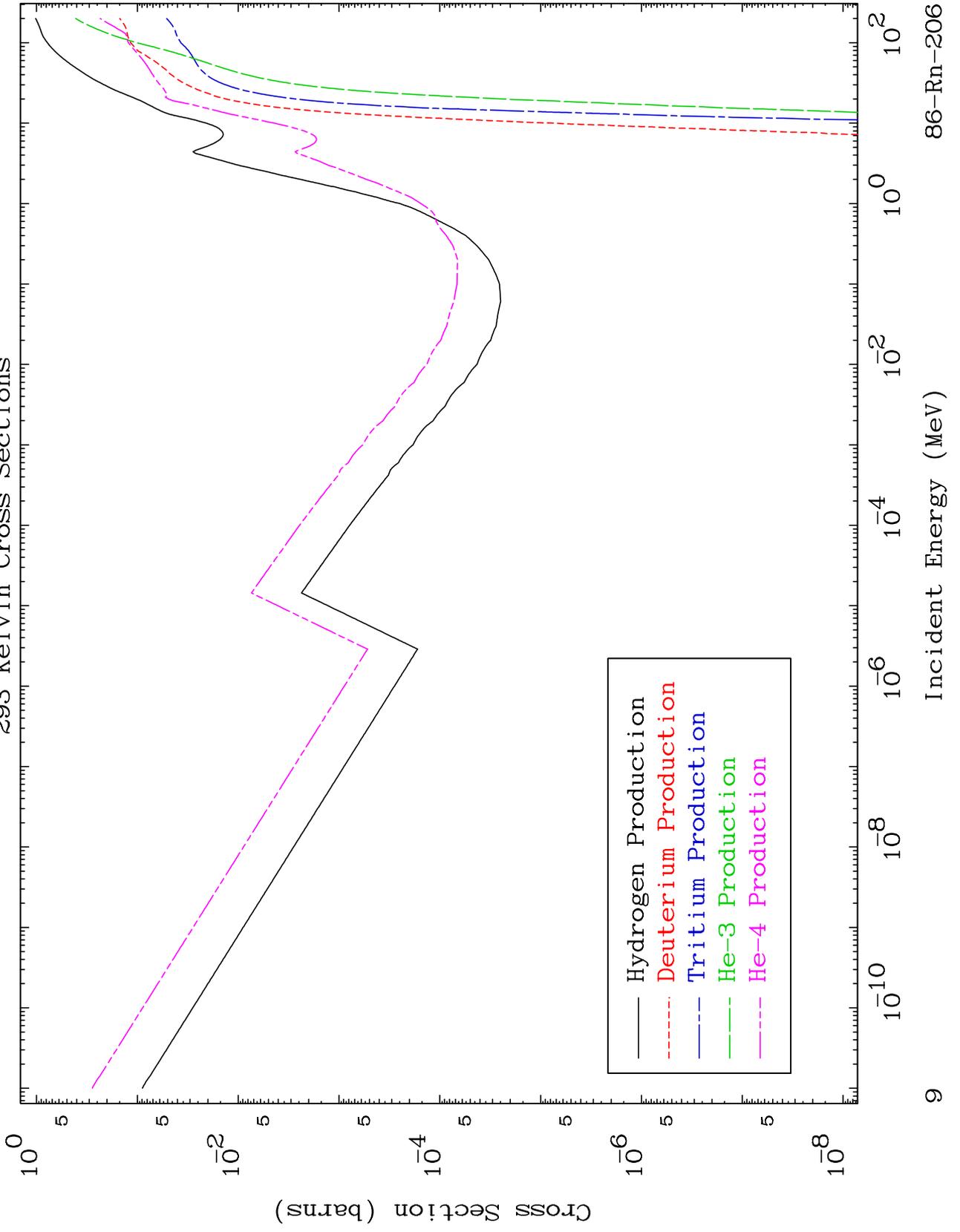


MAT 8610

Charged Particle  
293 Kelvin Cross Sections

86-Rn-206



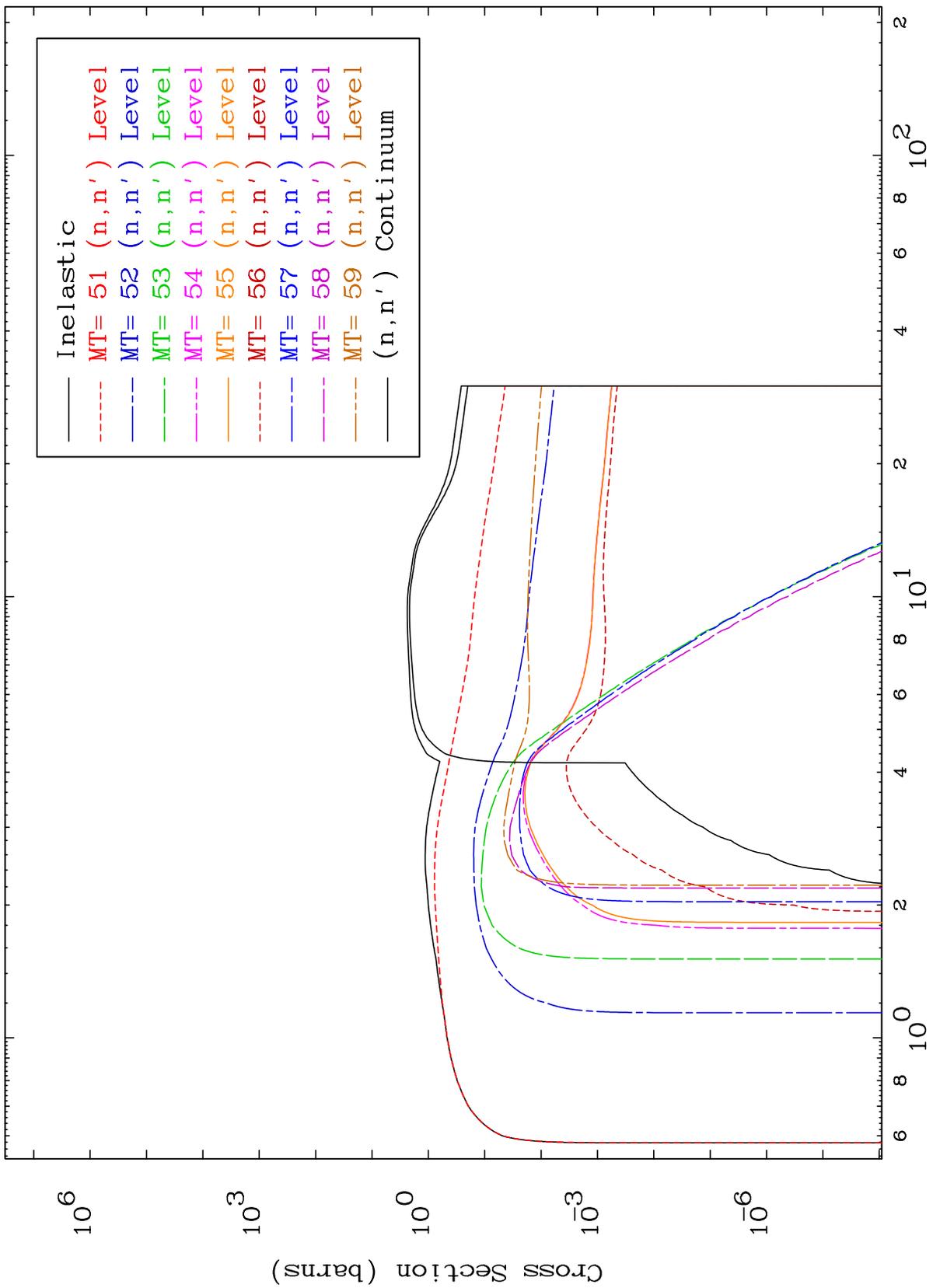


MAT 8610

(n,n') Levels

86-Rn-206

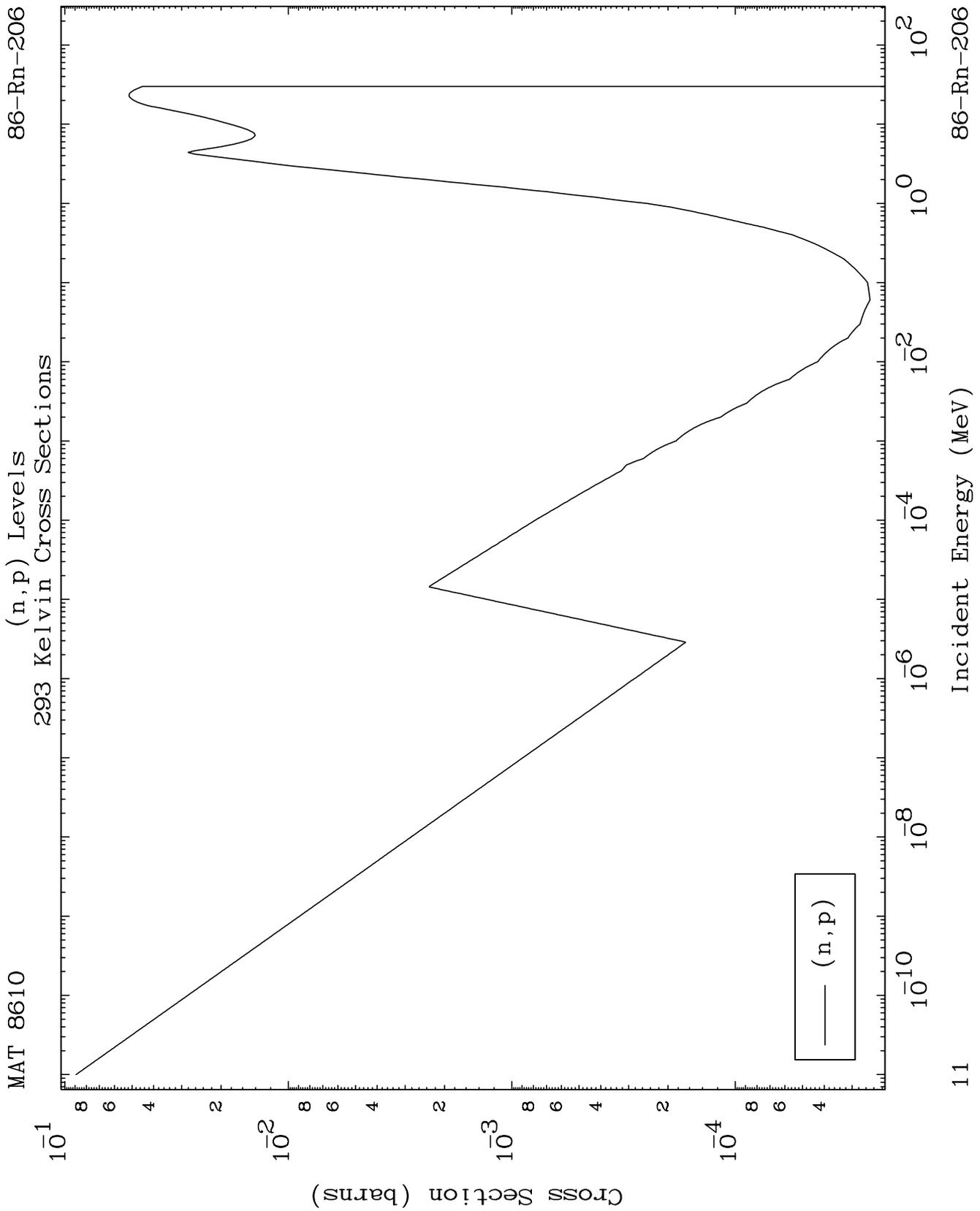
293 Kelvin Cross Sections



10

Incident Energy (MeV)

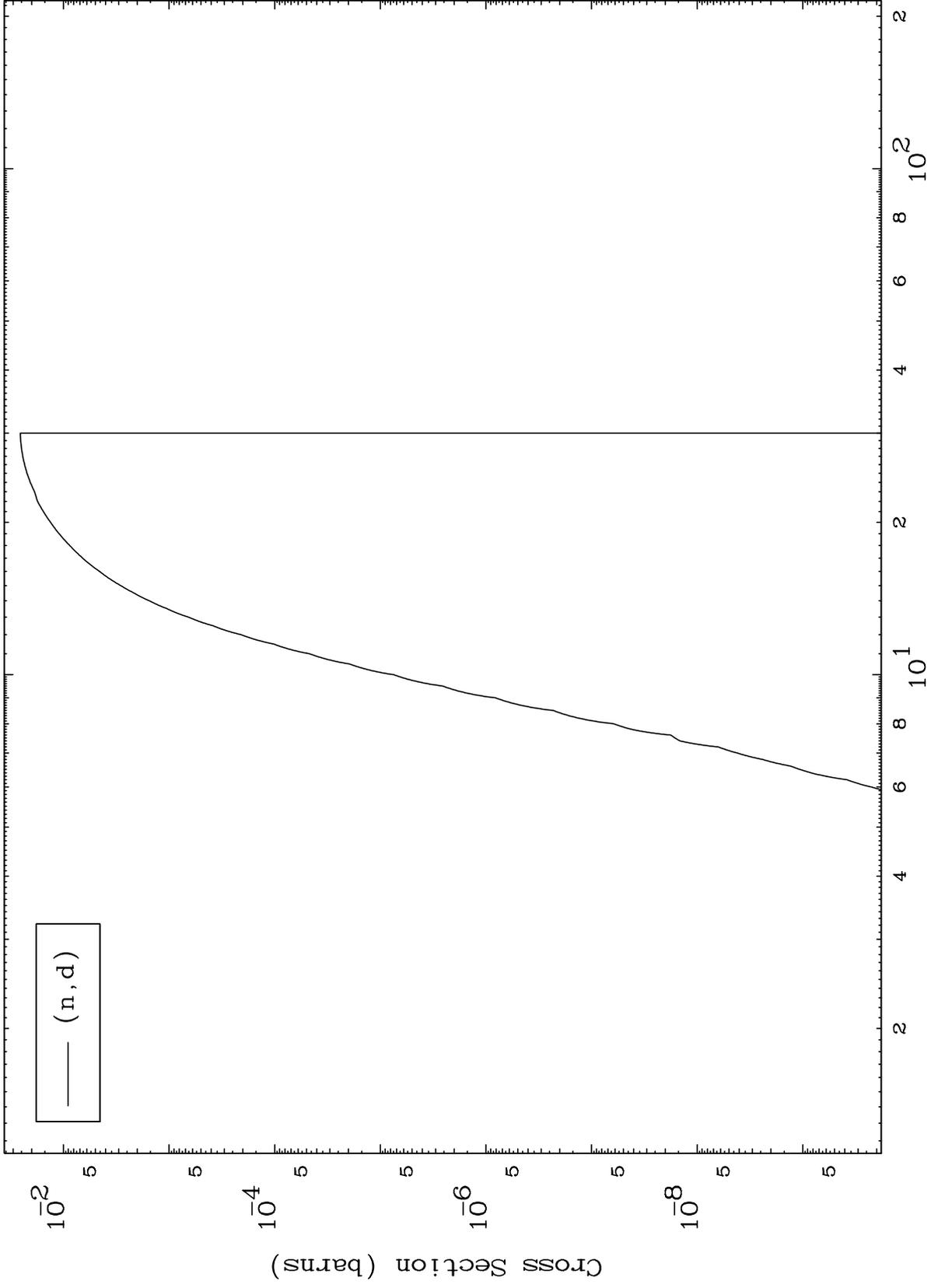
86-Rn-206



MAT 8610

(n,d) Levels  
293 Kelvin Cross Sections

86-Rn-206



12

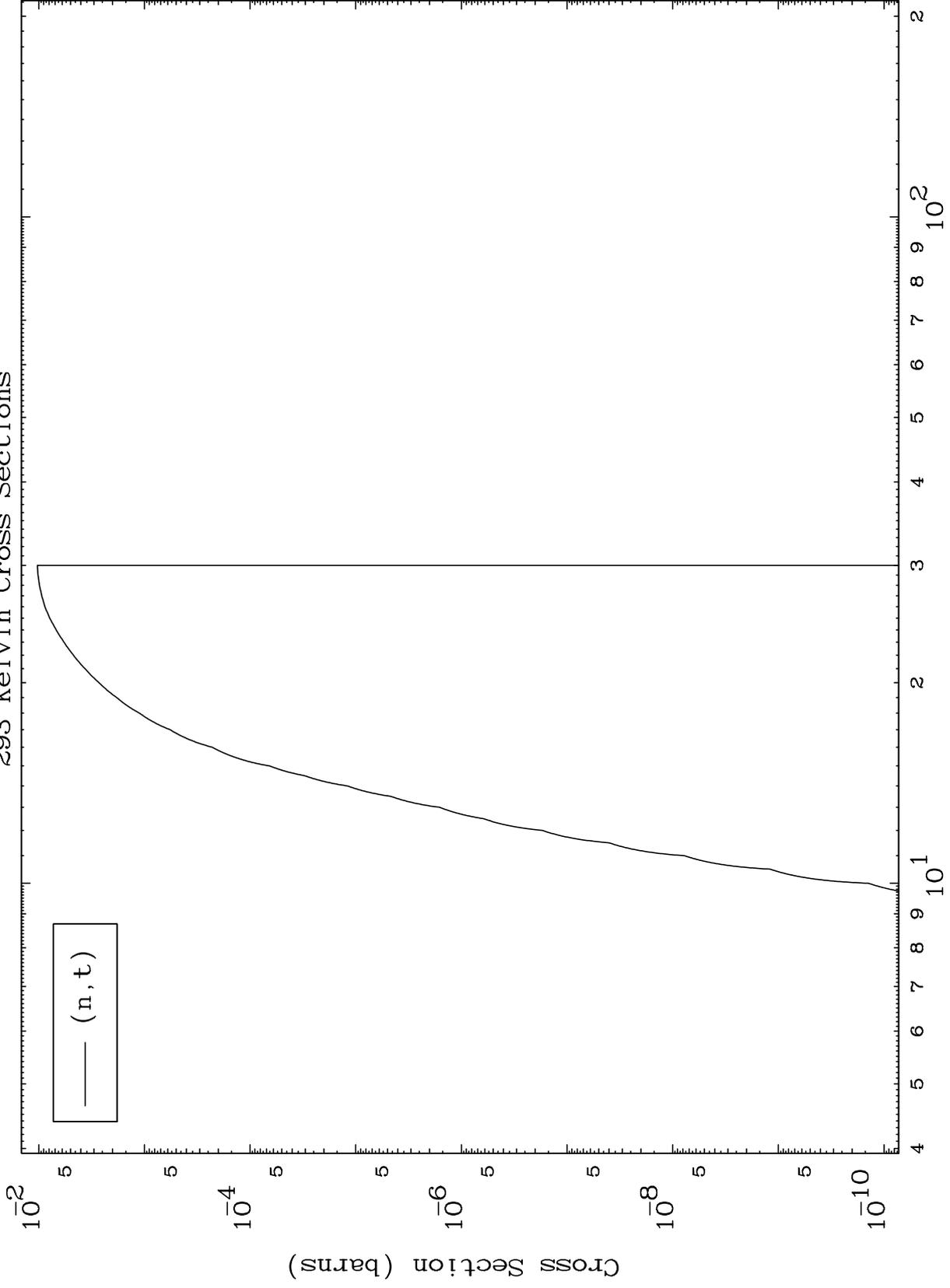
Incident Energy (MeV)

86-Rn-206

MAT 8610

(n,t) Levels  
293 Kelvin Cross Sections

86-Rn-206



13

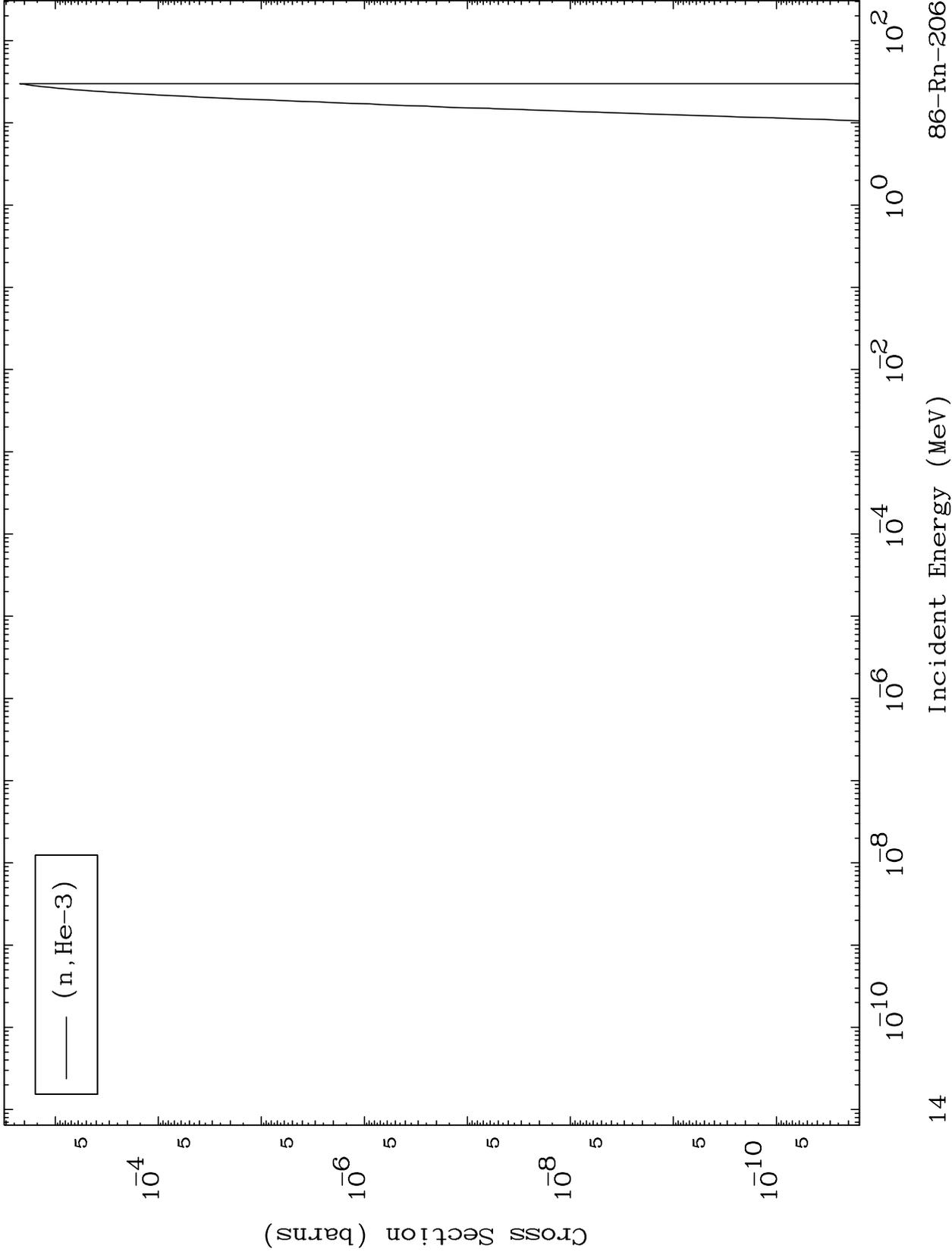
Incident Energy (MeV)

86-Rn-206

MAT 8610

(n,He3) Levels  
293 Kelvin Cross Sections

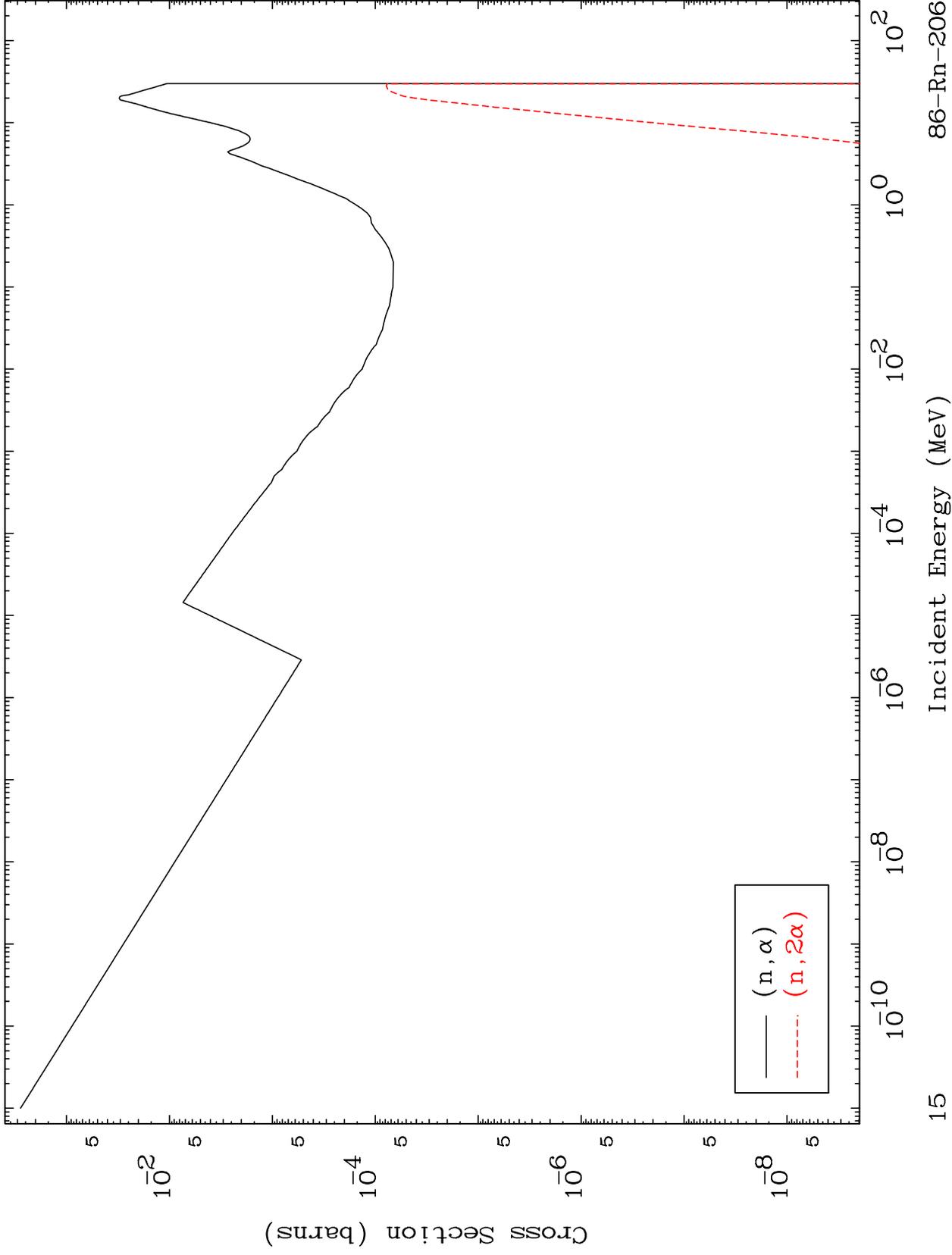
86-Rn-206



MAT 8610

$(n, \alpha)$  Levels  
293 Kelvin Cross Sections

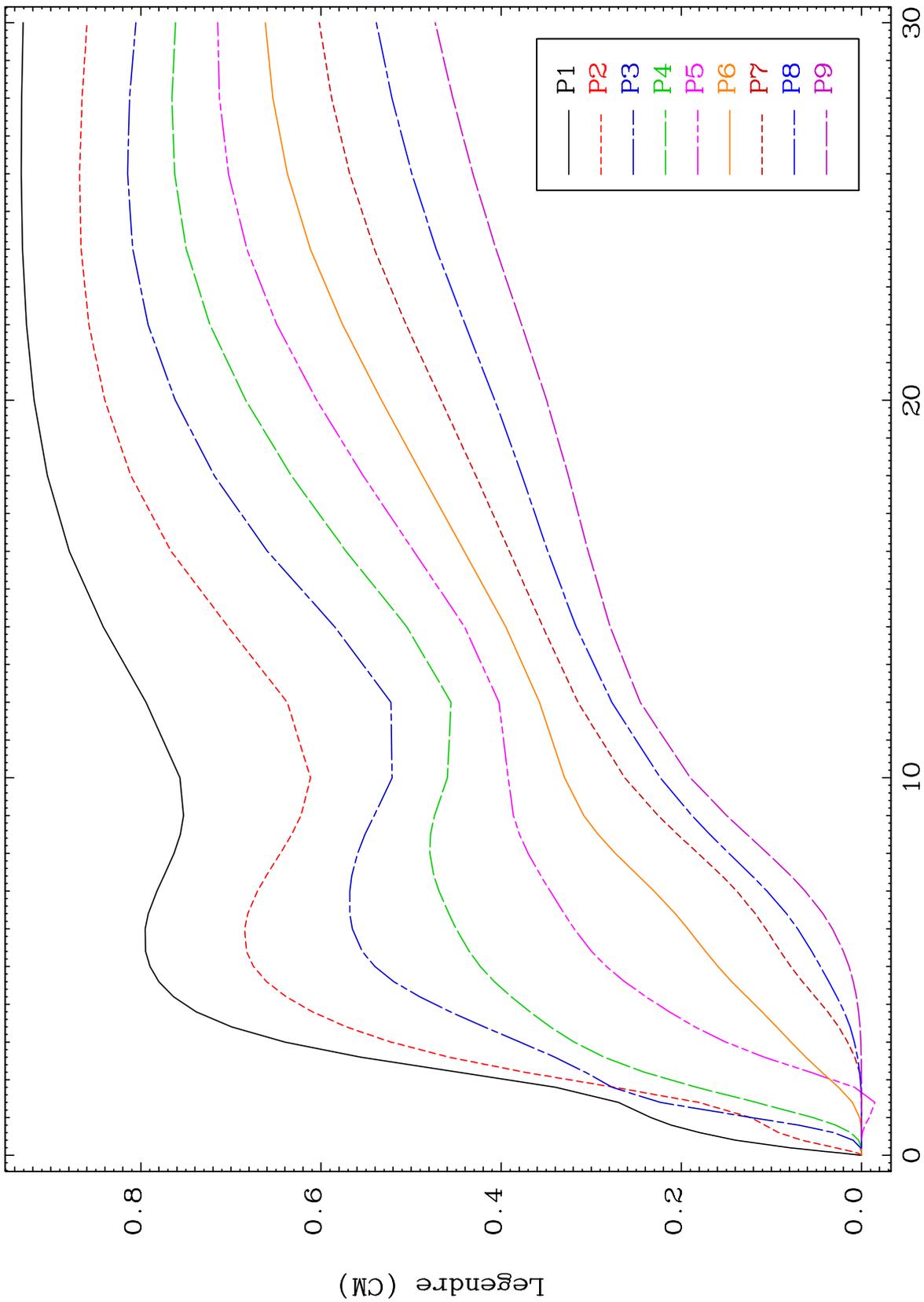
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MAT 8610

86-Rn-206

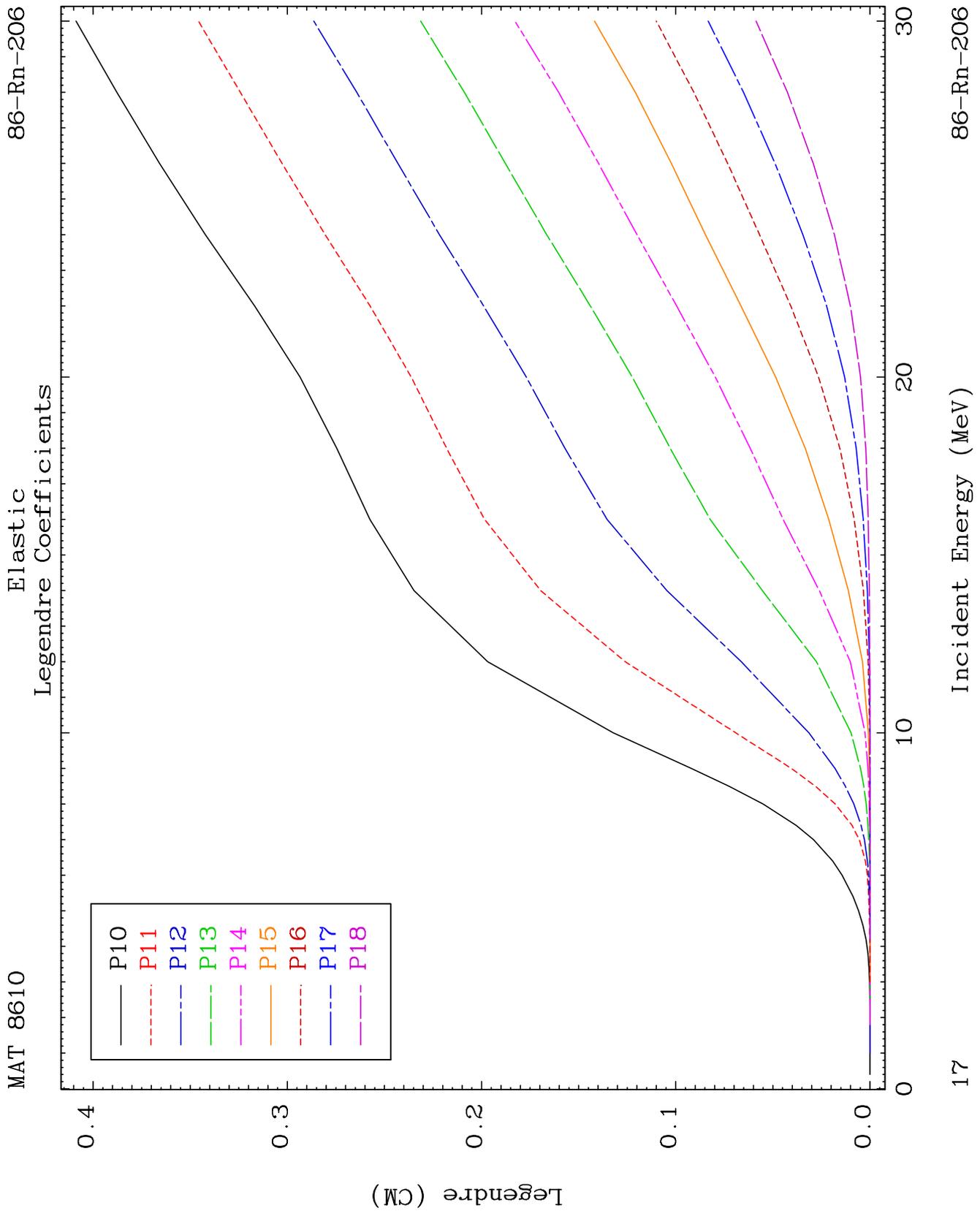
Elastic Legendre Coefficients



16

Incident Energy (MeV)

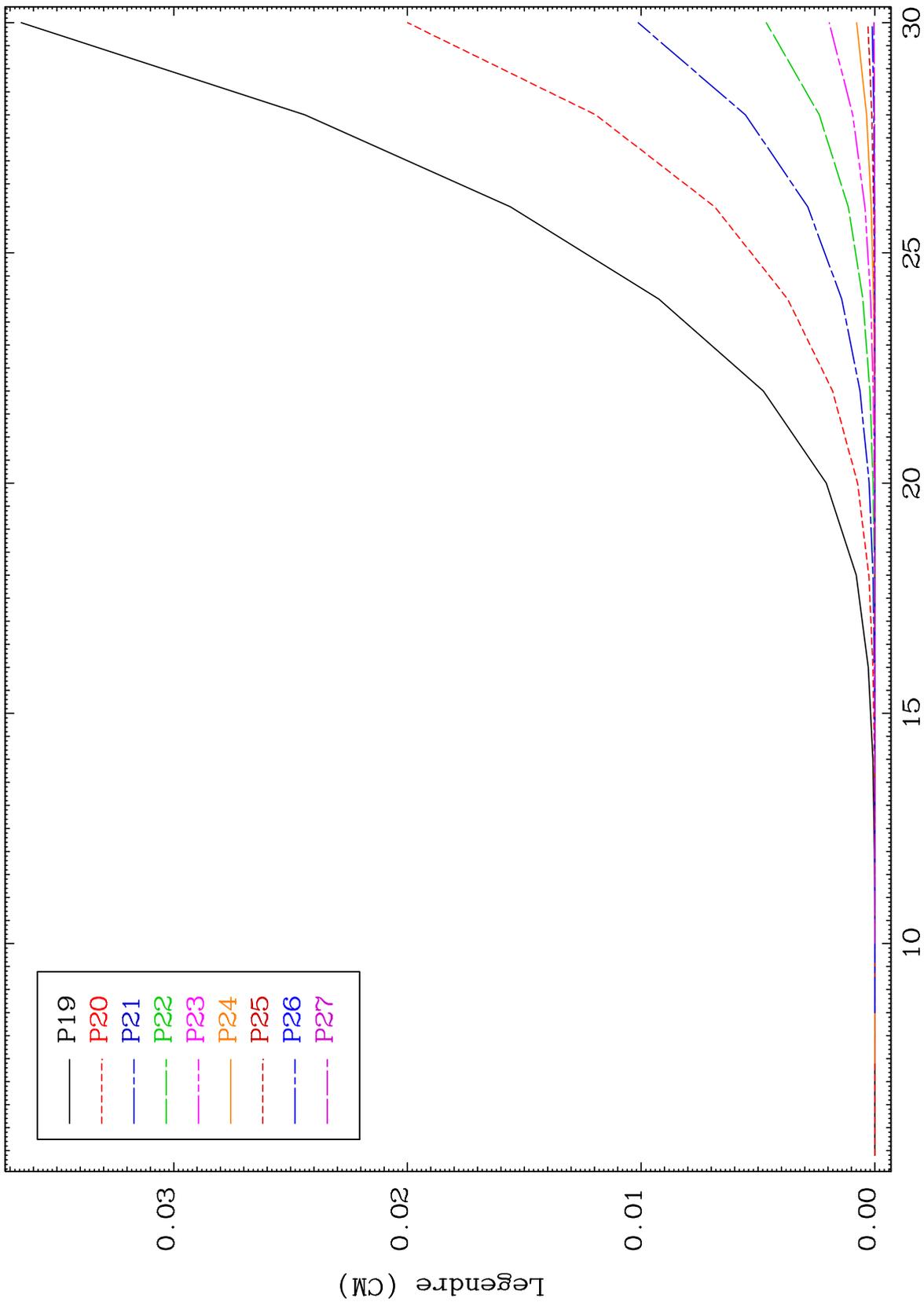
86-Rn-206



MAT 8610

Elastic  
Legendre Coefficients

86-Rn-206



18

Incident Energy (MeV)

86-Rn-206

30

25

20

15

10

Legendre (CM)

0.03

0.02

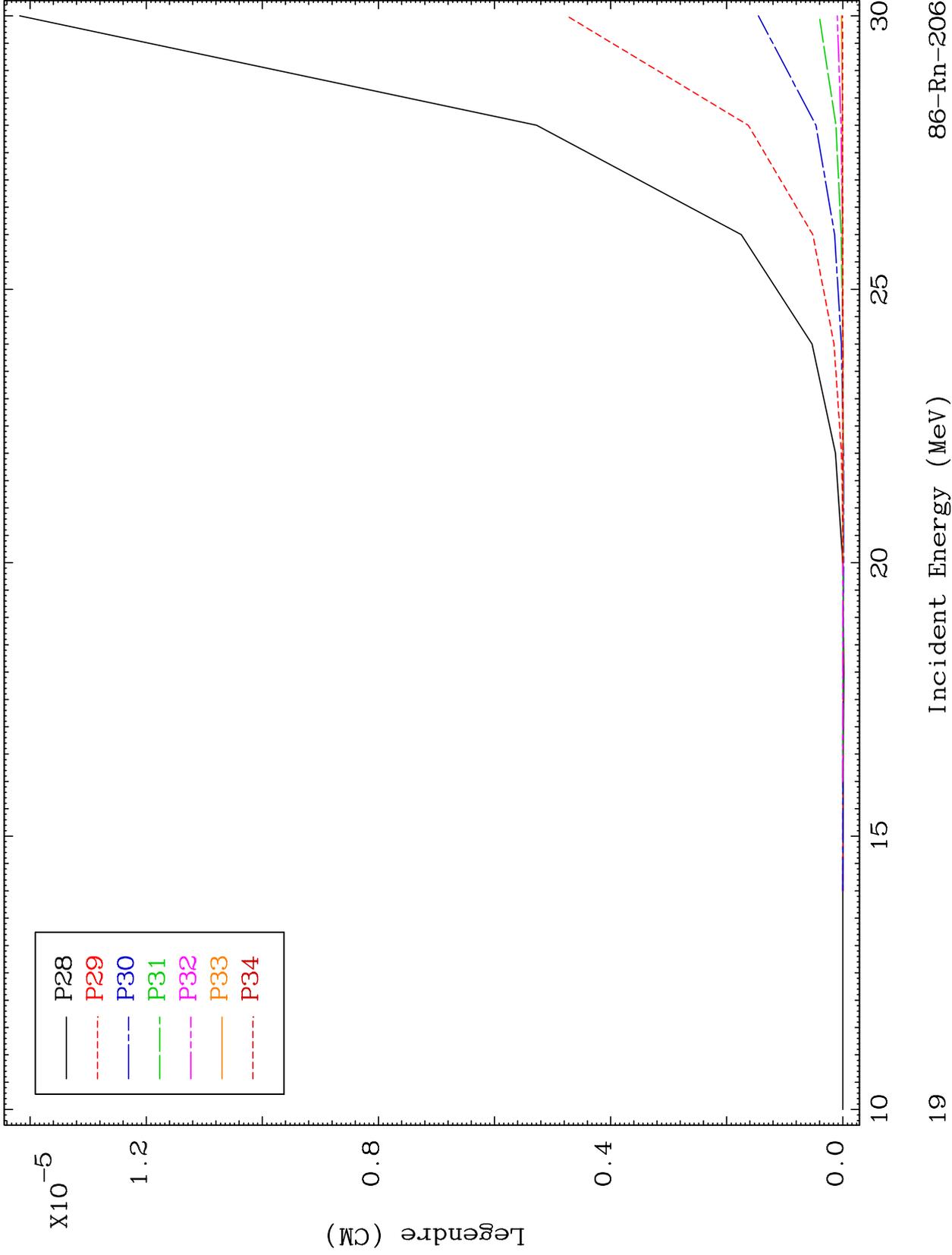
0.01

0.00

MAT 8610

Elastic Legendre Coefficients

86-Rn-206



86-Rn-206

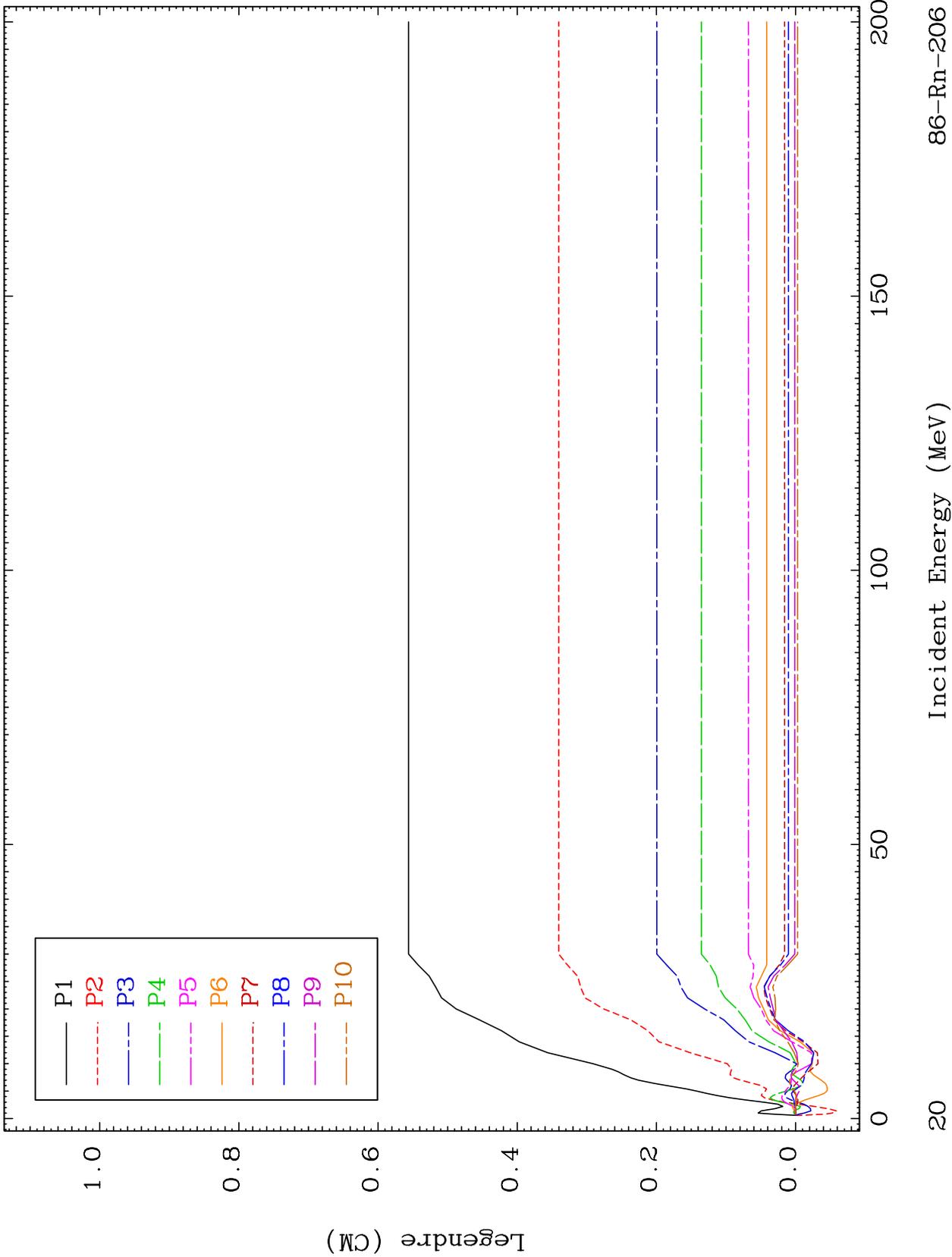
Incident Energy (MeV)

19

MAT 8610

MT= 51 (n,n') Level  
Legendre Coefficients

86-Rn-206



86-Rn-206

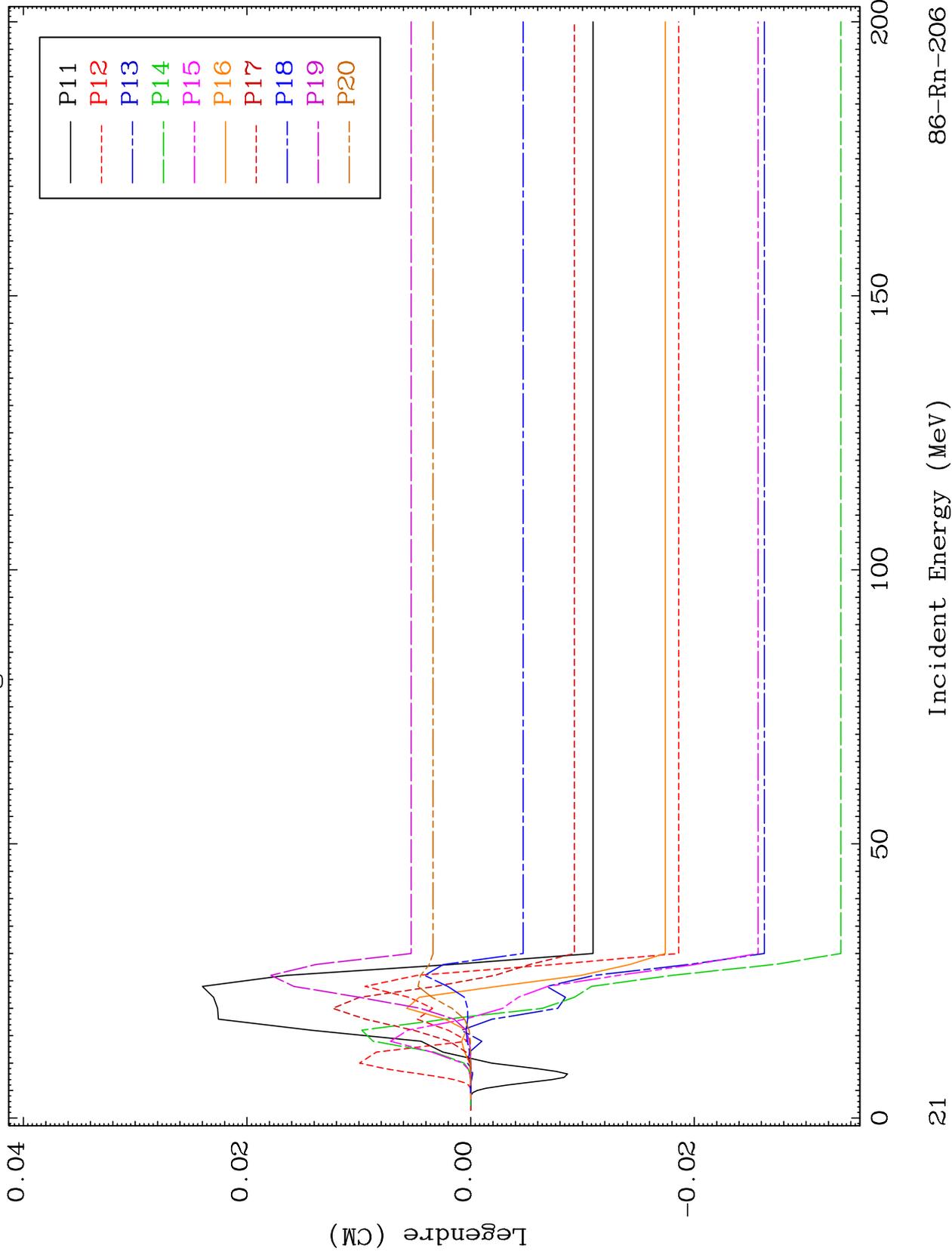
Incident Energy (MeV)

Legendre (CM)

MAT 8610

MT= 51 (n,n') Level  
Legendre Coefficients

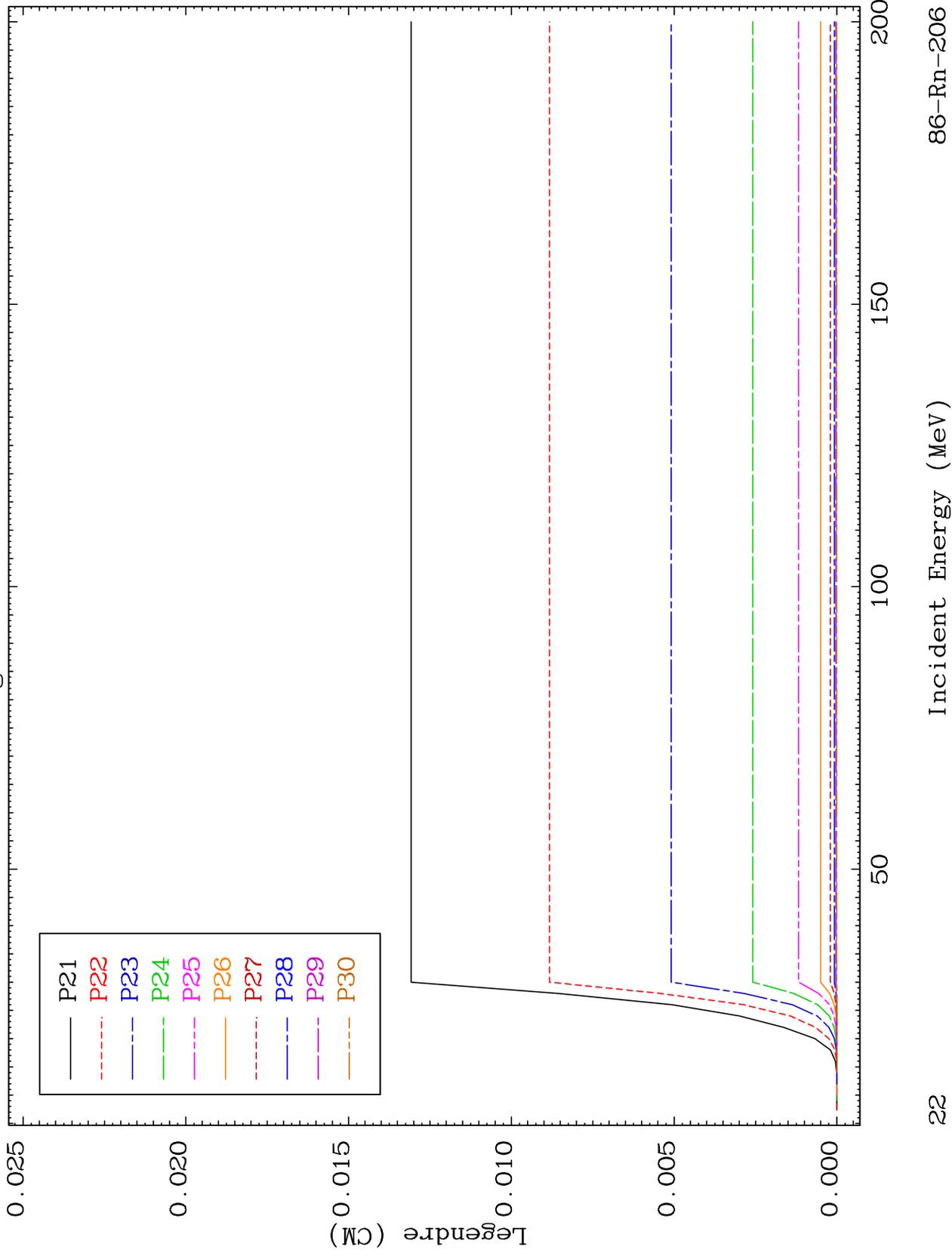
86-Rn-206



21

Incident Energy (MeV)

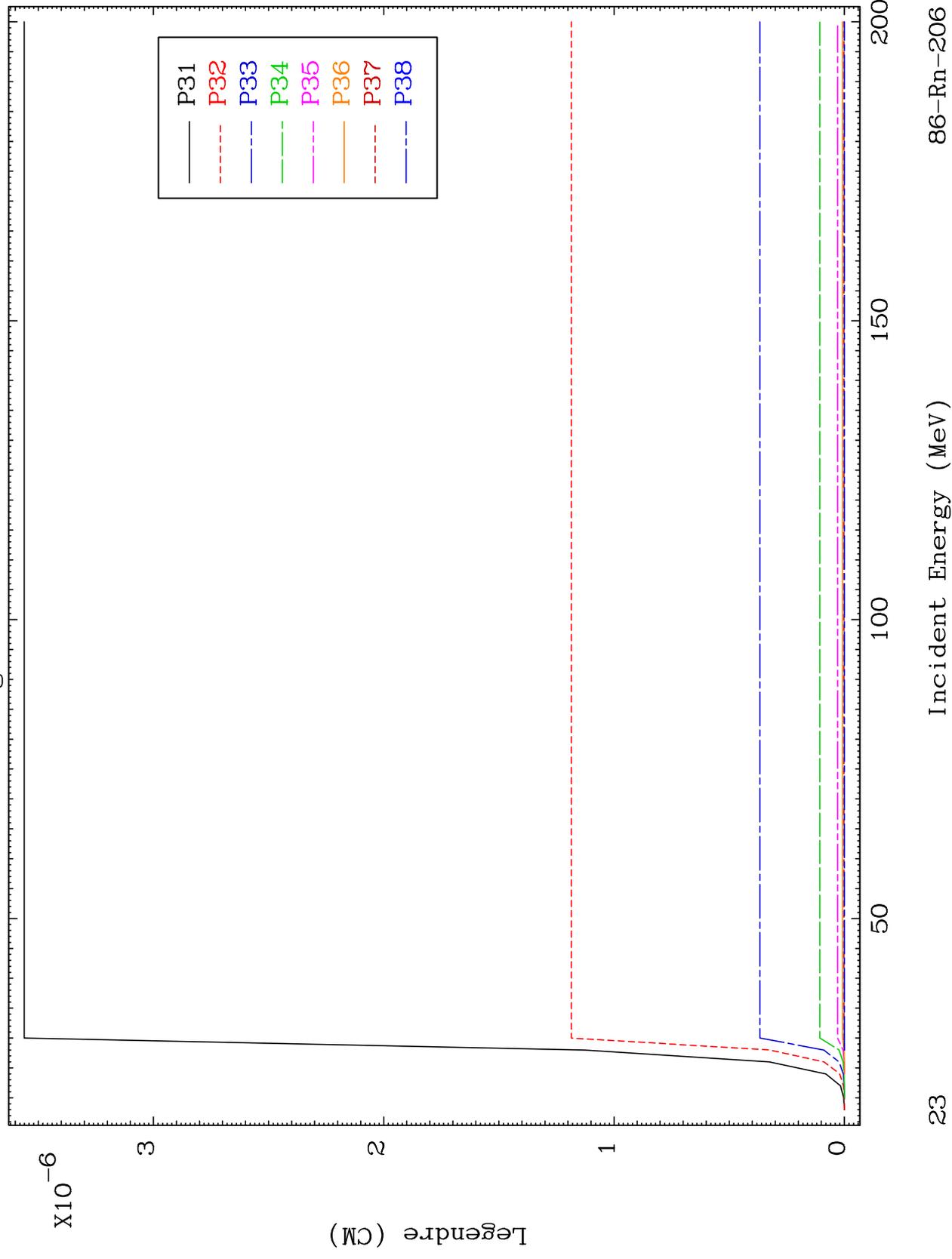
86-Rn-206



MAT 8610

MT= 51 (n,n') Level  
Legendre Coefficients

86-Rn-206



23

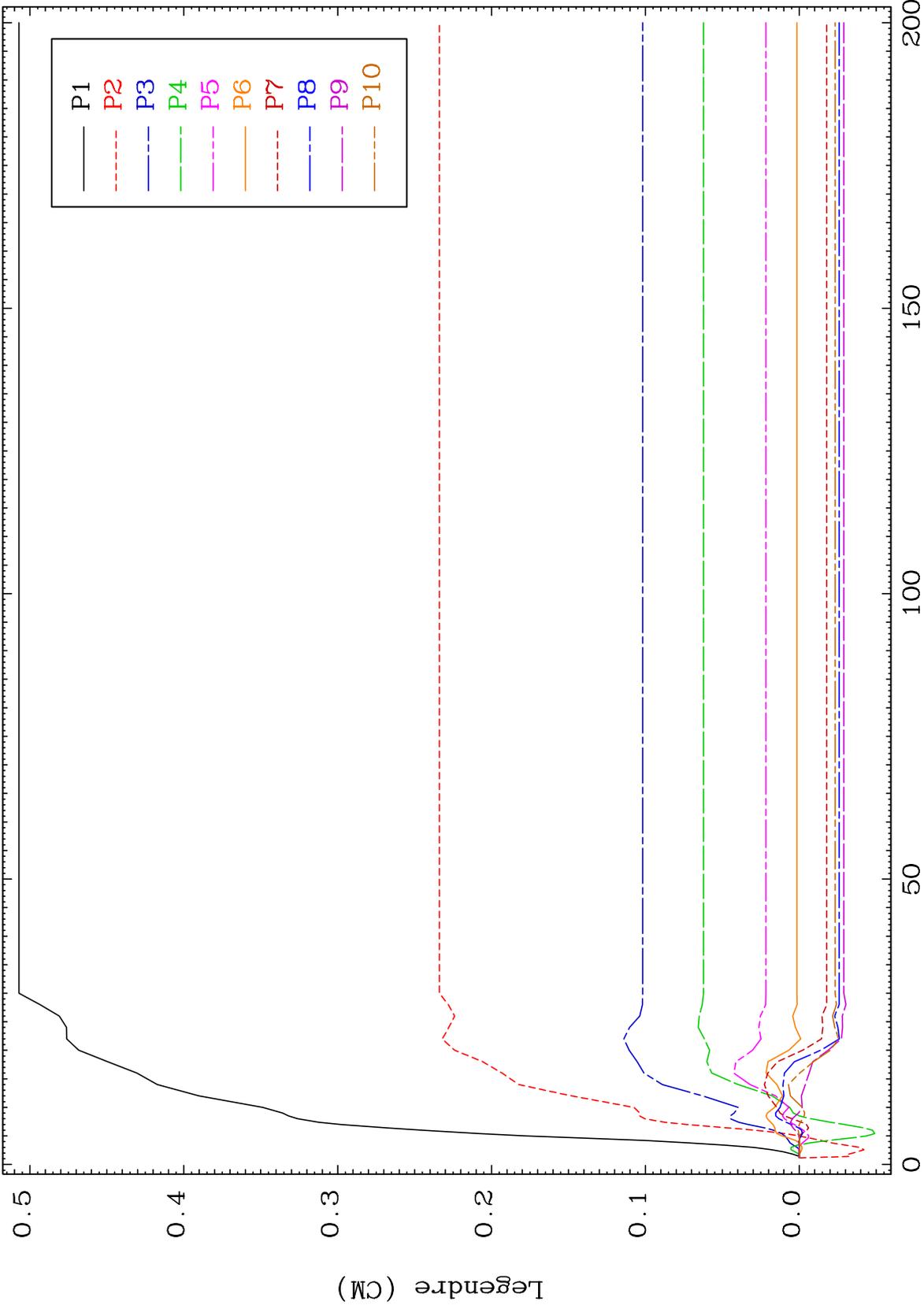
86-Rn-206

MAT 8610

MT= 52 (n,n') Level

86-Rn-206

Legendre Coefficients



24

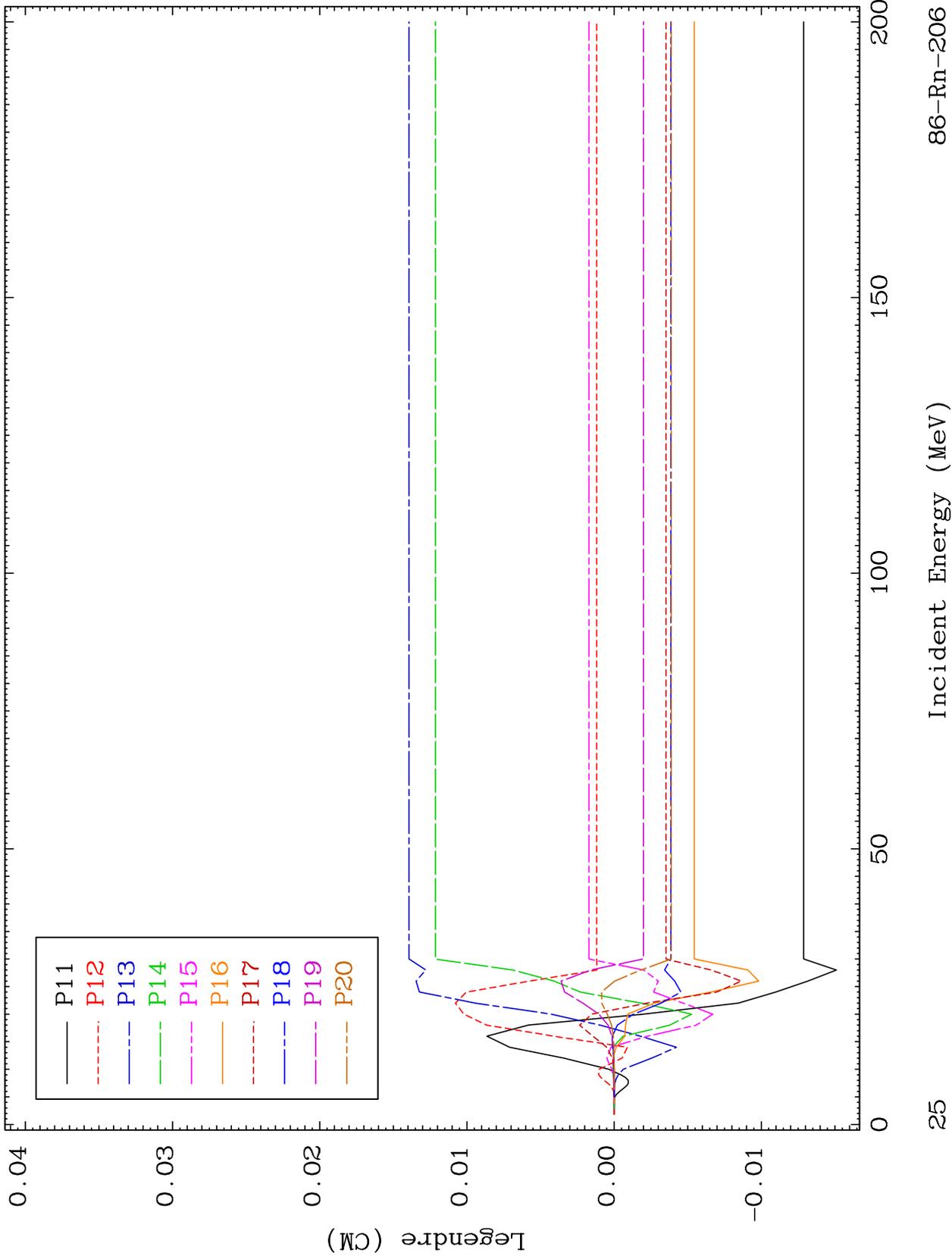
Incident Energy (MeV)

86-Rn-206

MAT 8610

MT= 52 (n,n') Level  
Legendre Coefficients

86-Rn-206



86-Rn-206

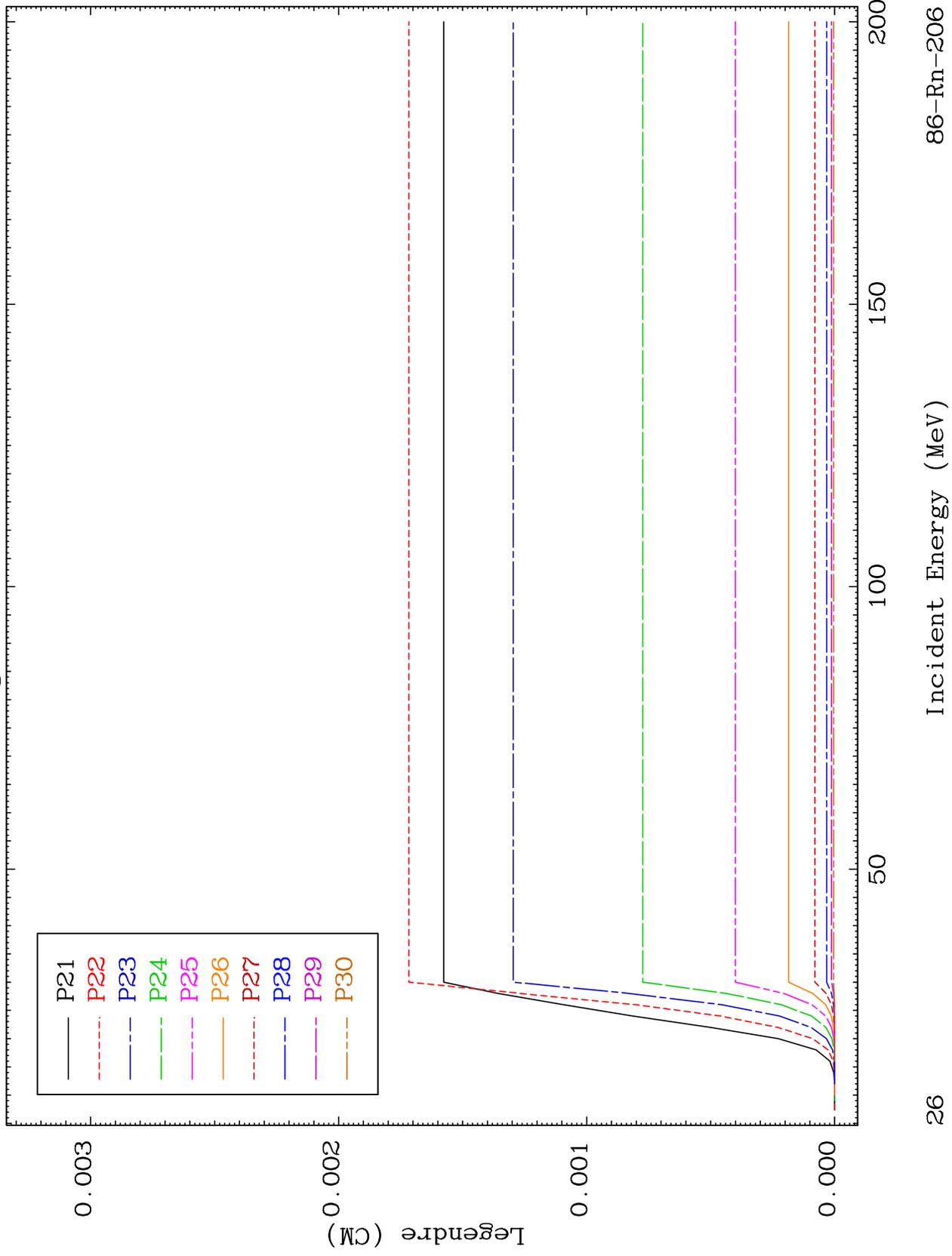
Incident Energy (MeV)

25

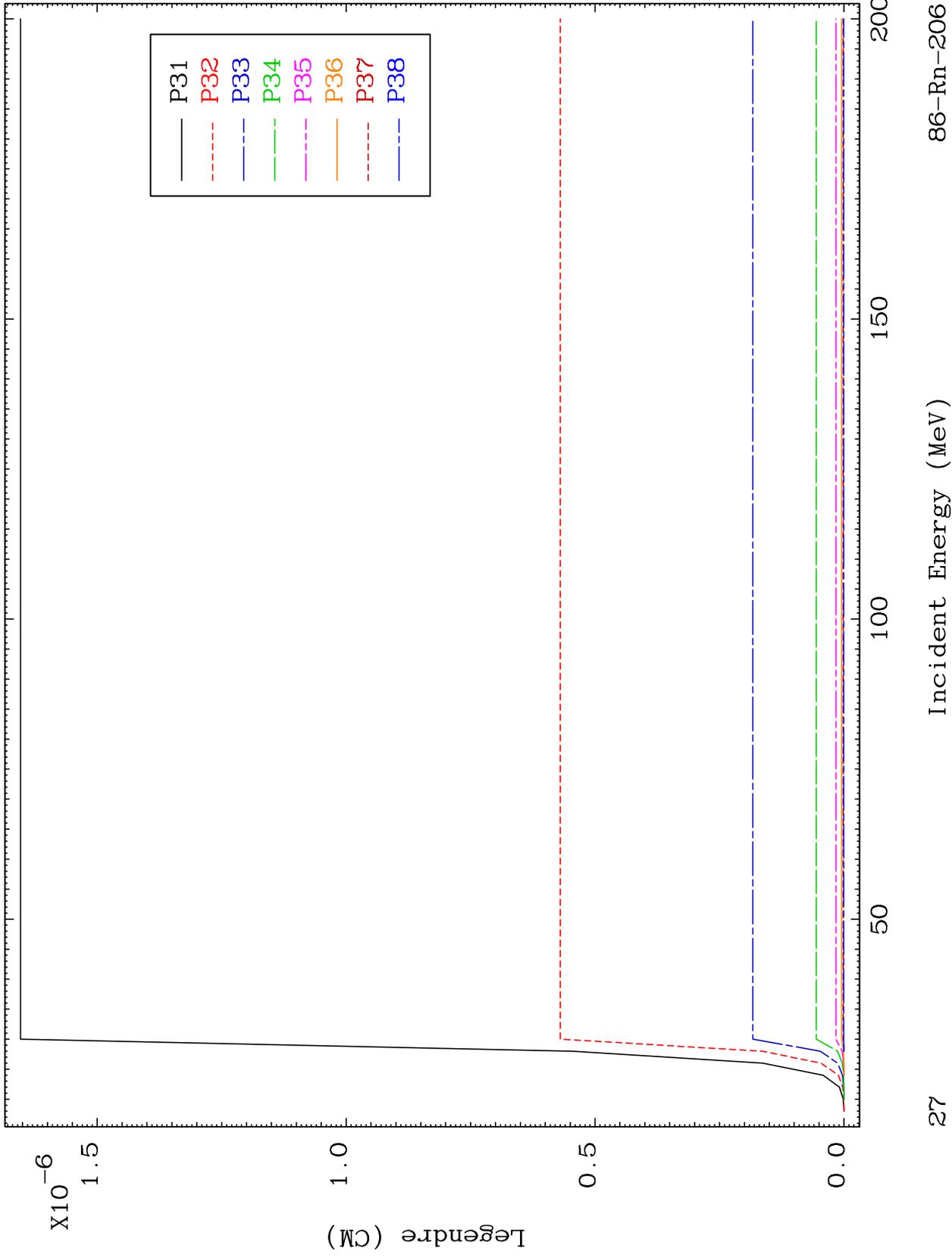
MAT 8610

MT= 52 (n,n') Level  
Legendre Coefficients

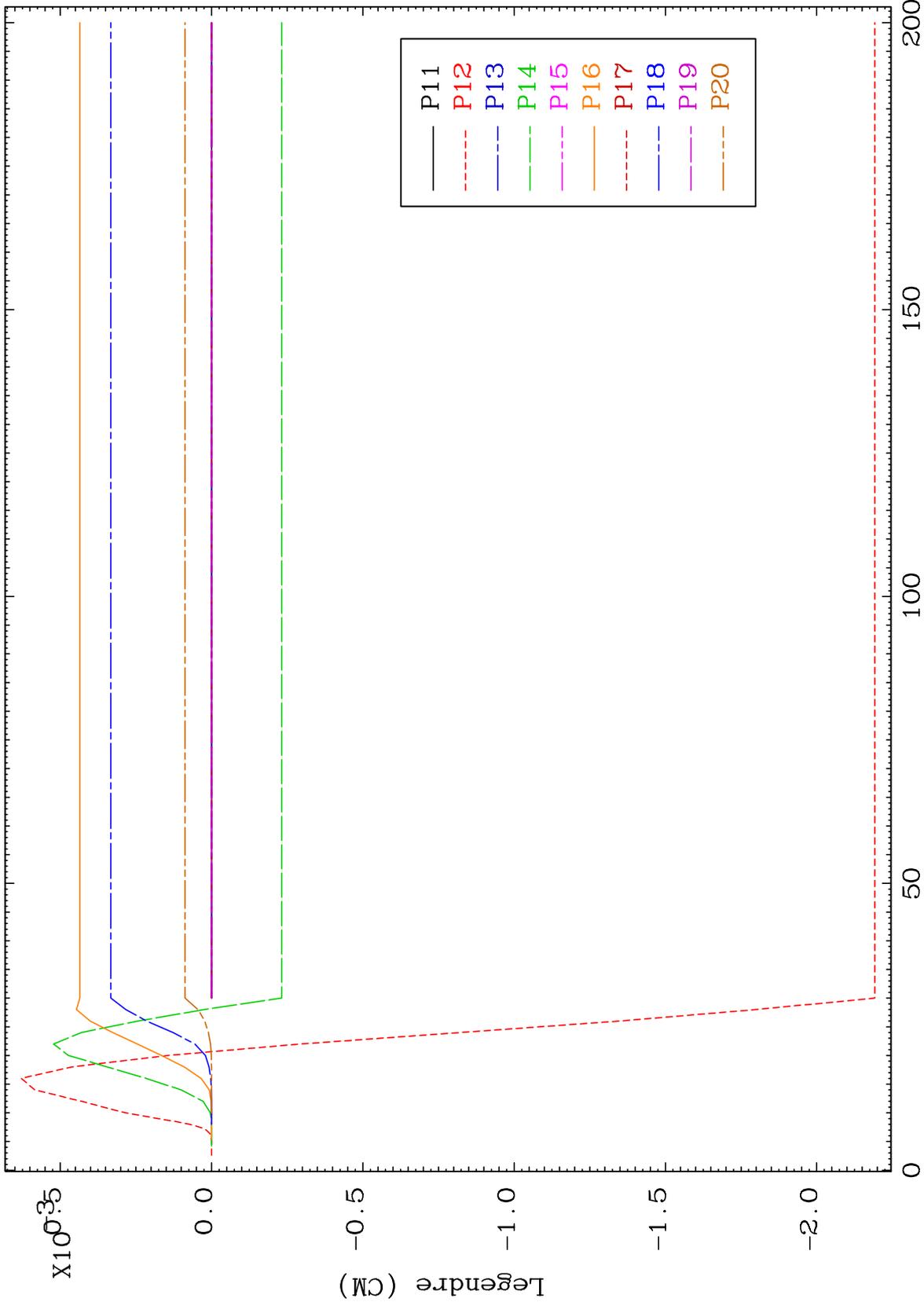
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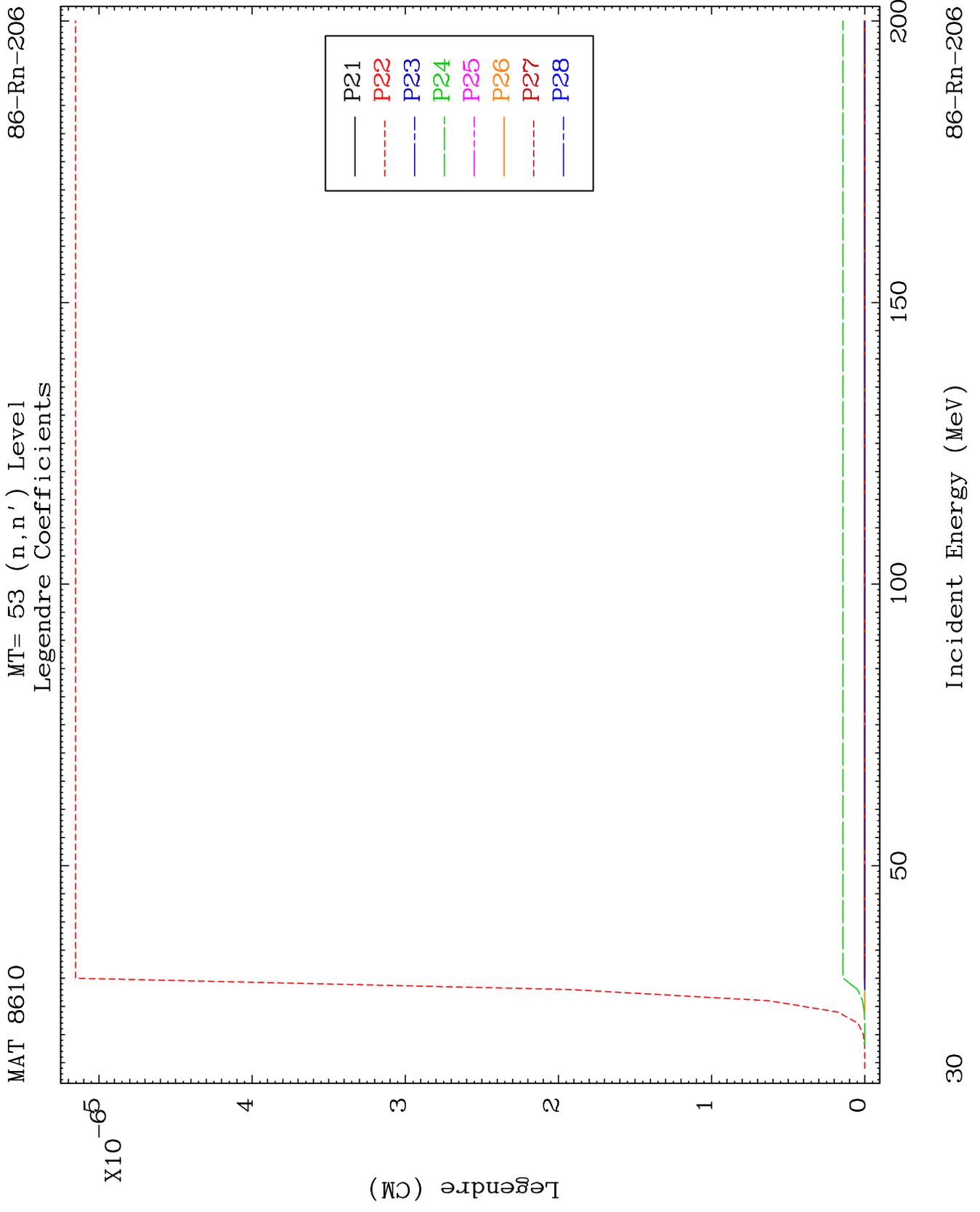


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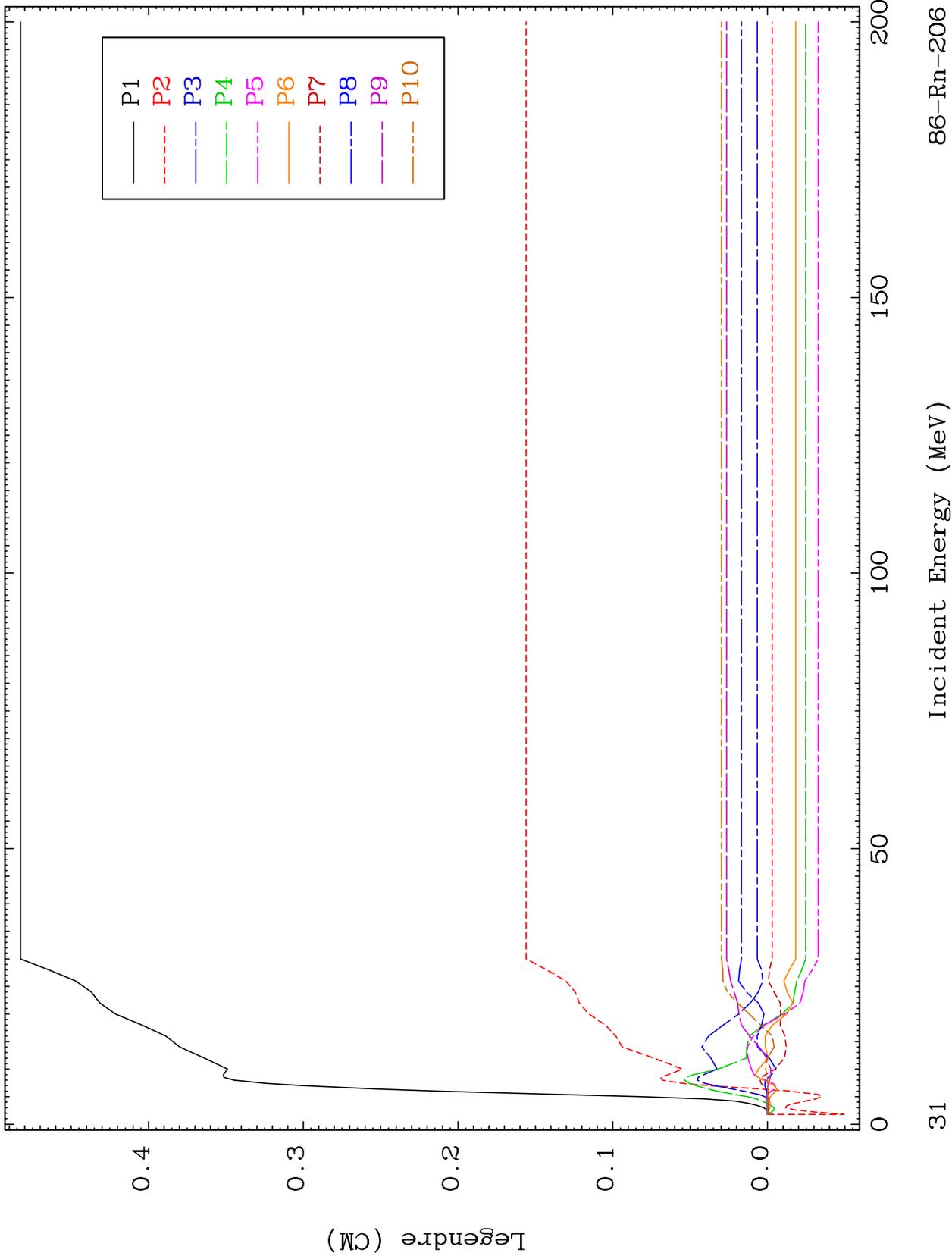




MAT 8610

MT= 54 (n,n') Level  
Legendre Coefficients

86-Rn-206



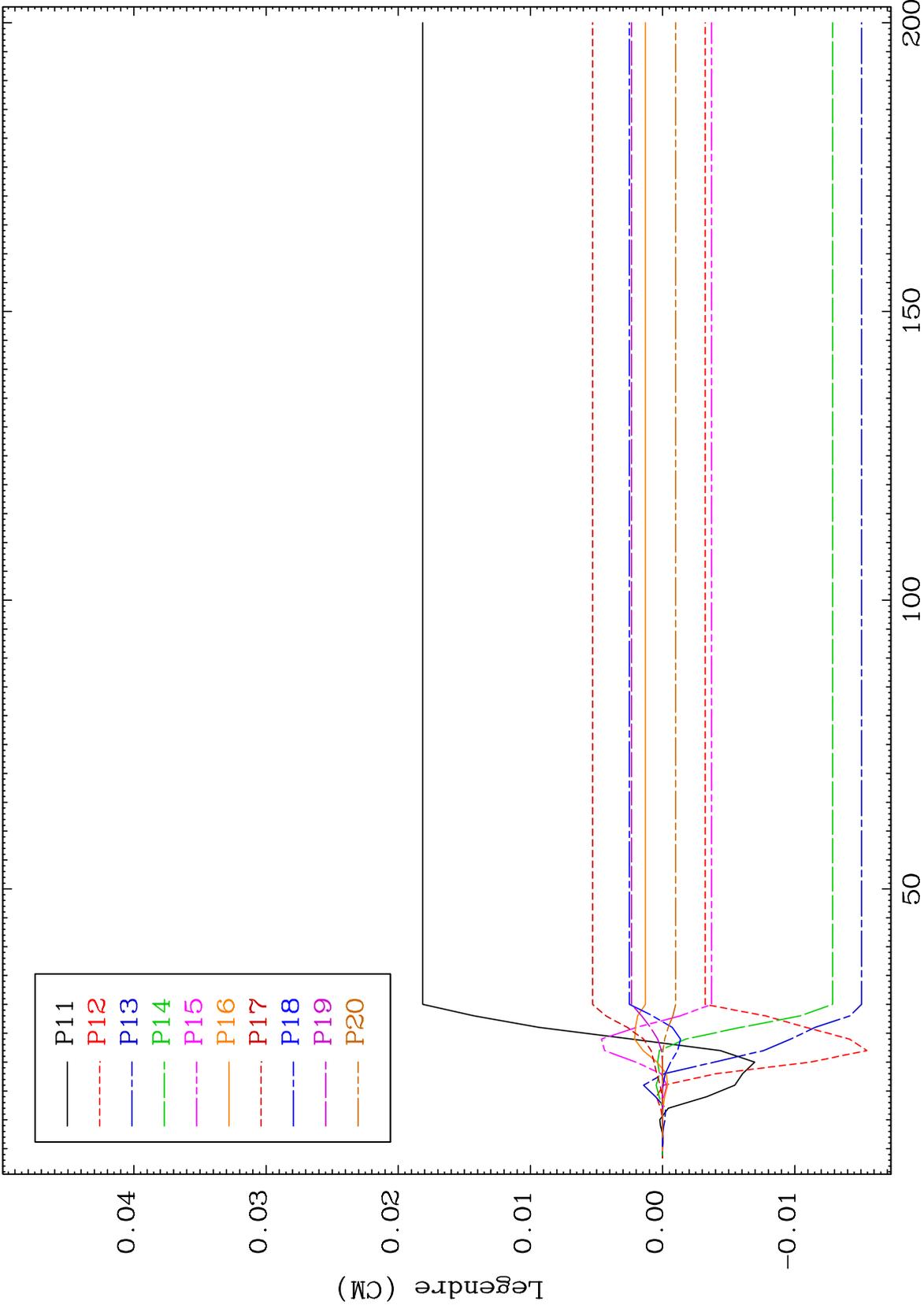
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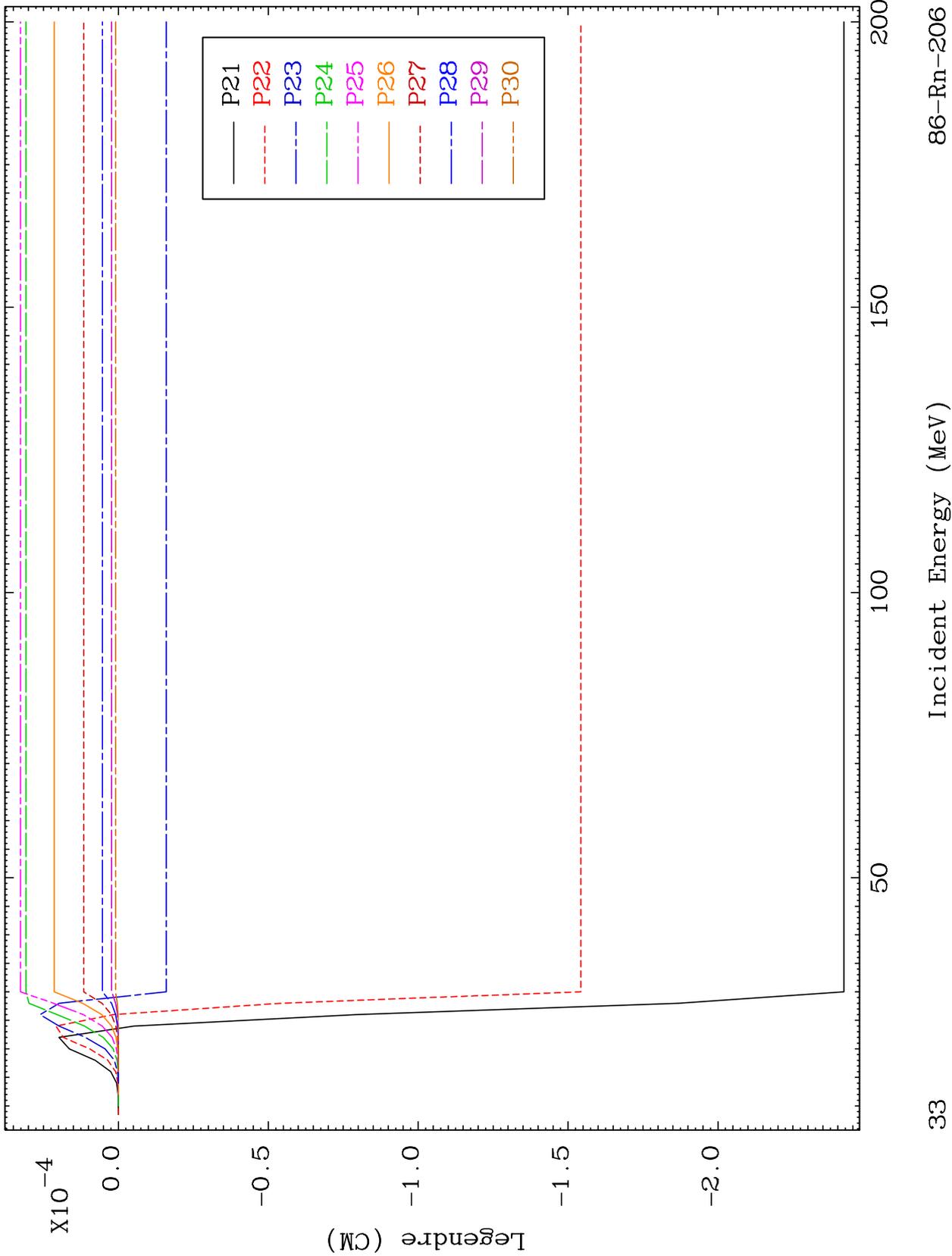
86-Rn-206

MAT 8610

MT= 54 (n,n') Level  
Legendre Coefficients

86-Rn-206

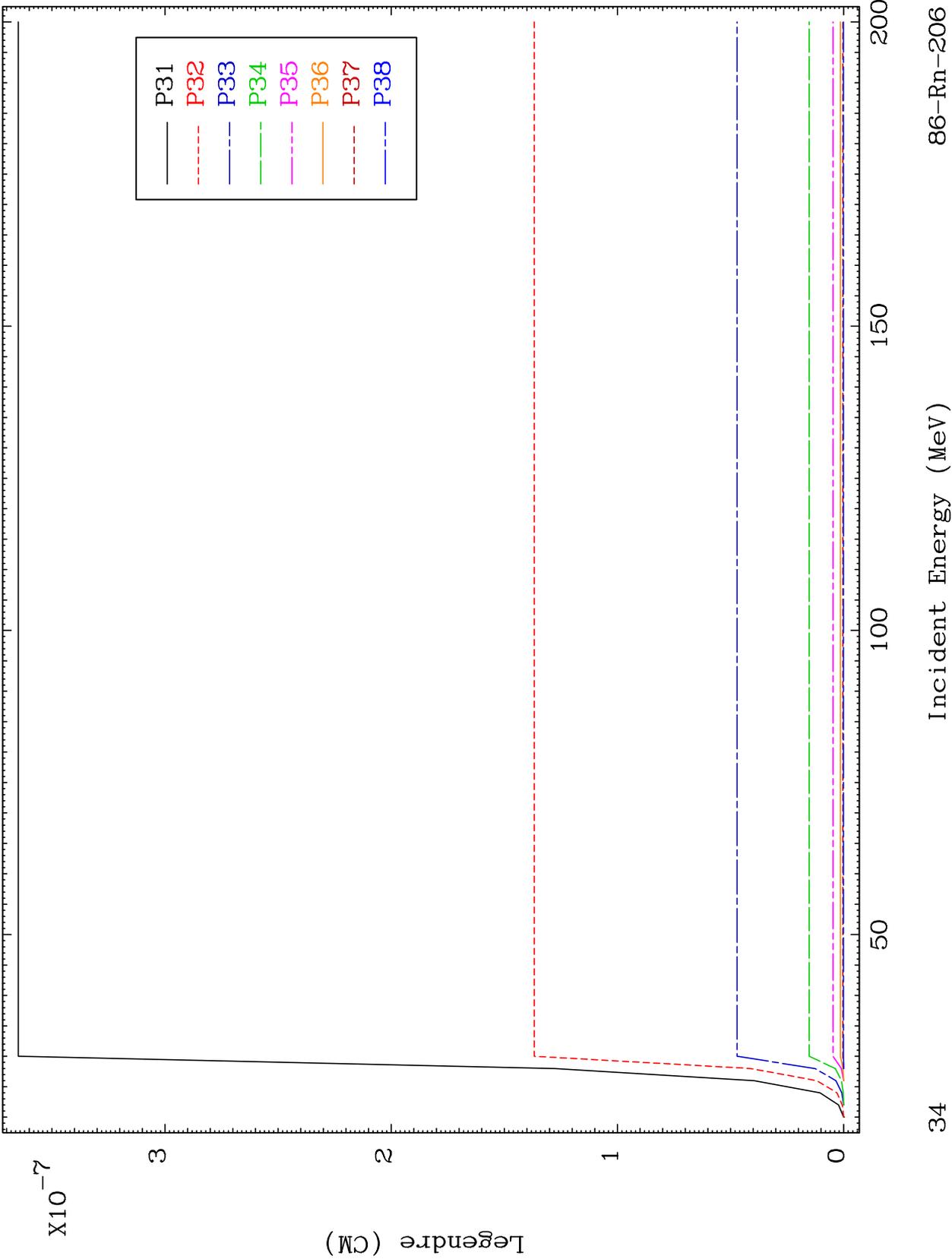




MAT 8610

MT= 54 (n,n') Level  
Legendre Coefficients

86-Rn-206

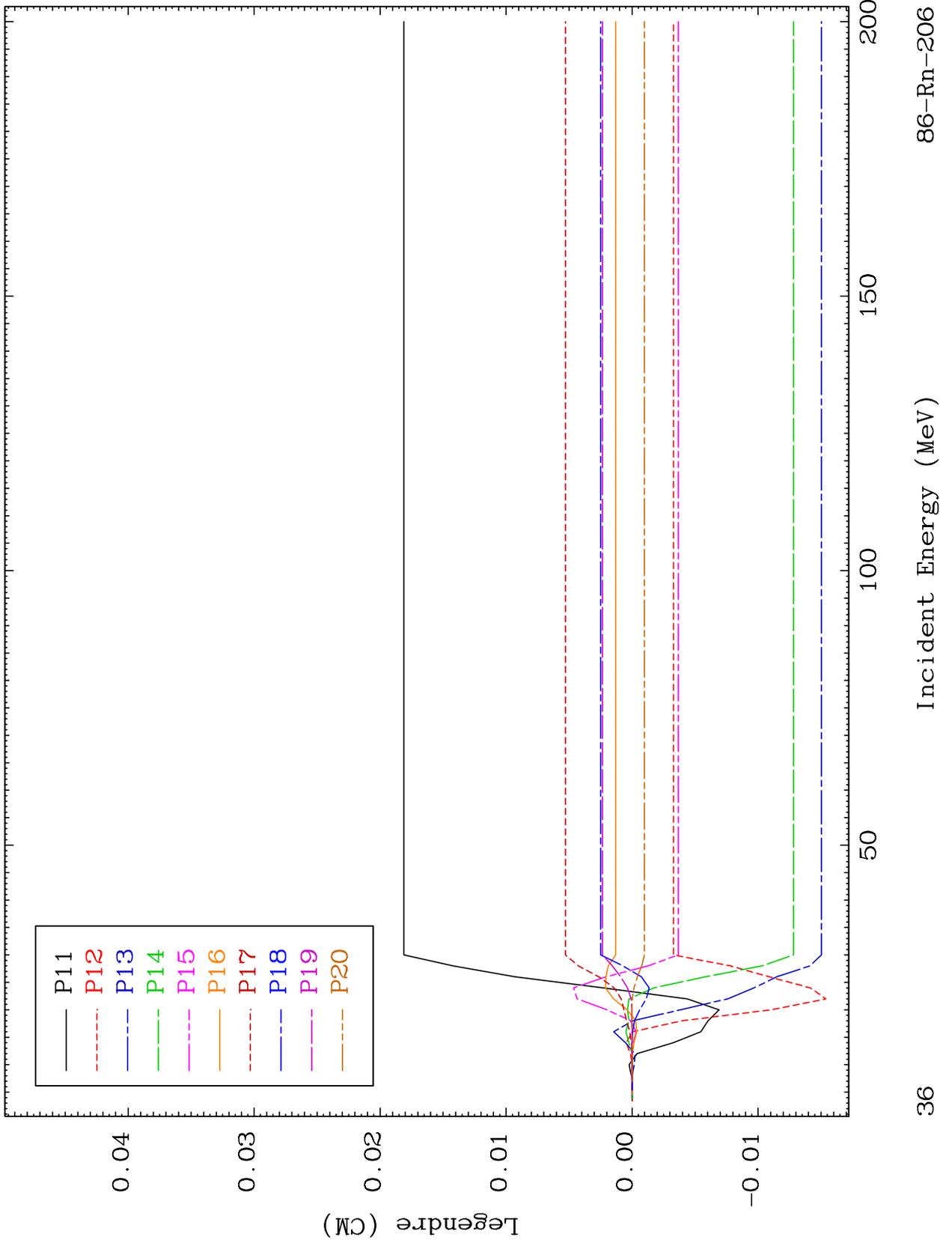


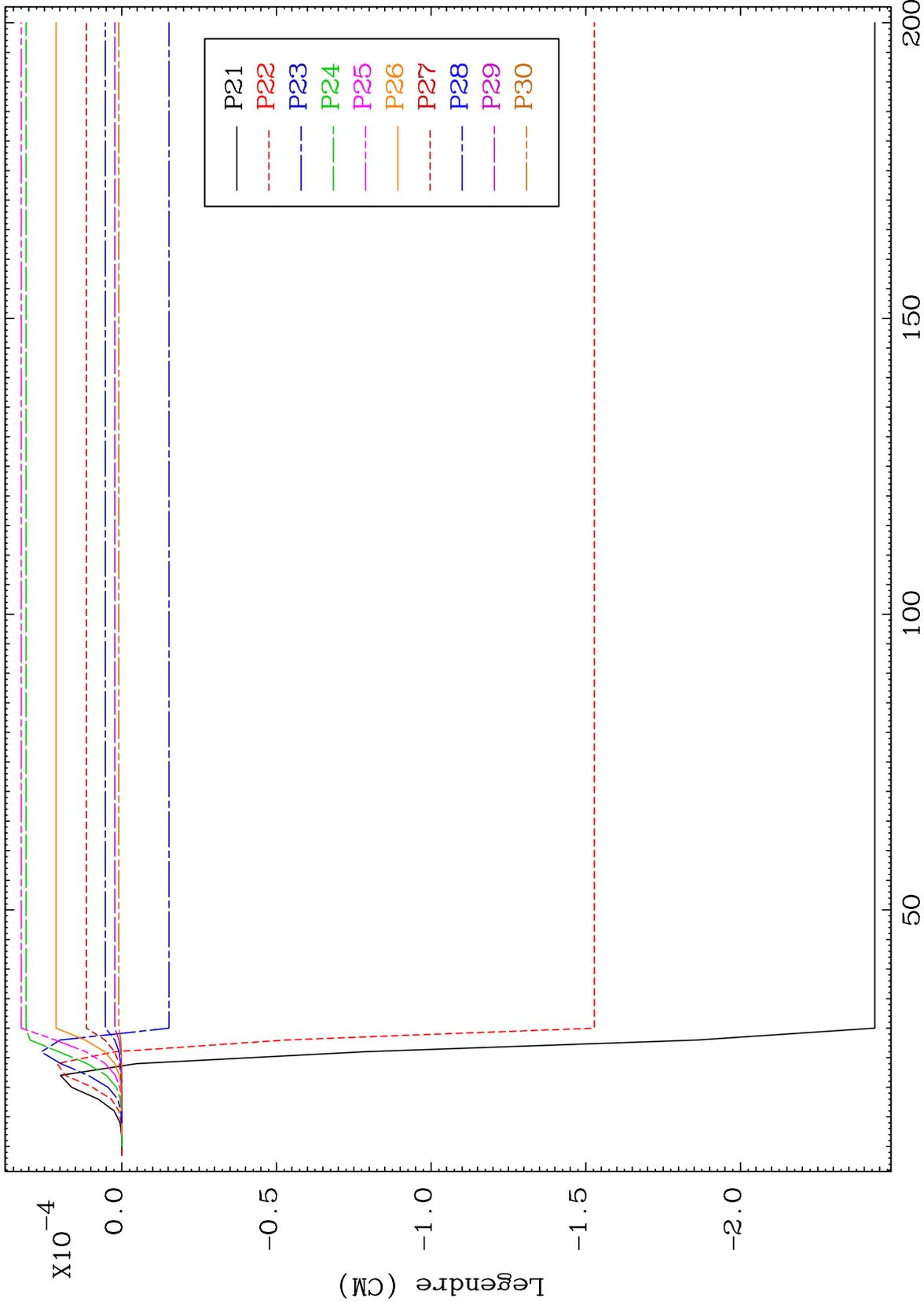
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Incident Energy (MeV)

86-Rn-206



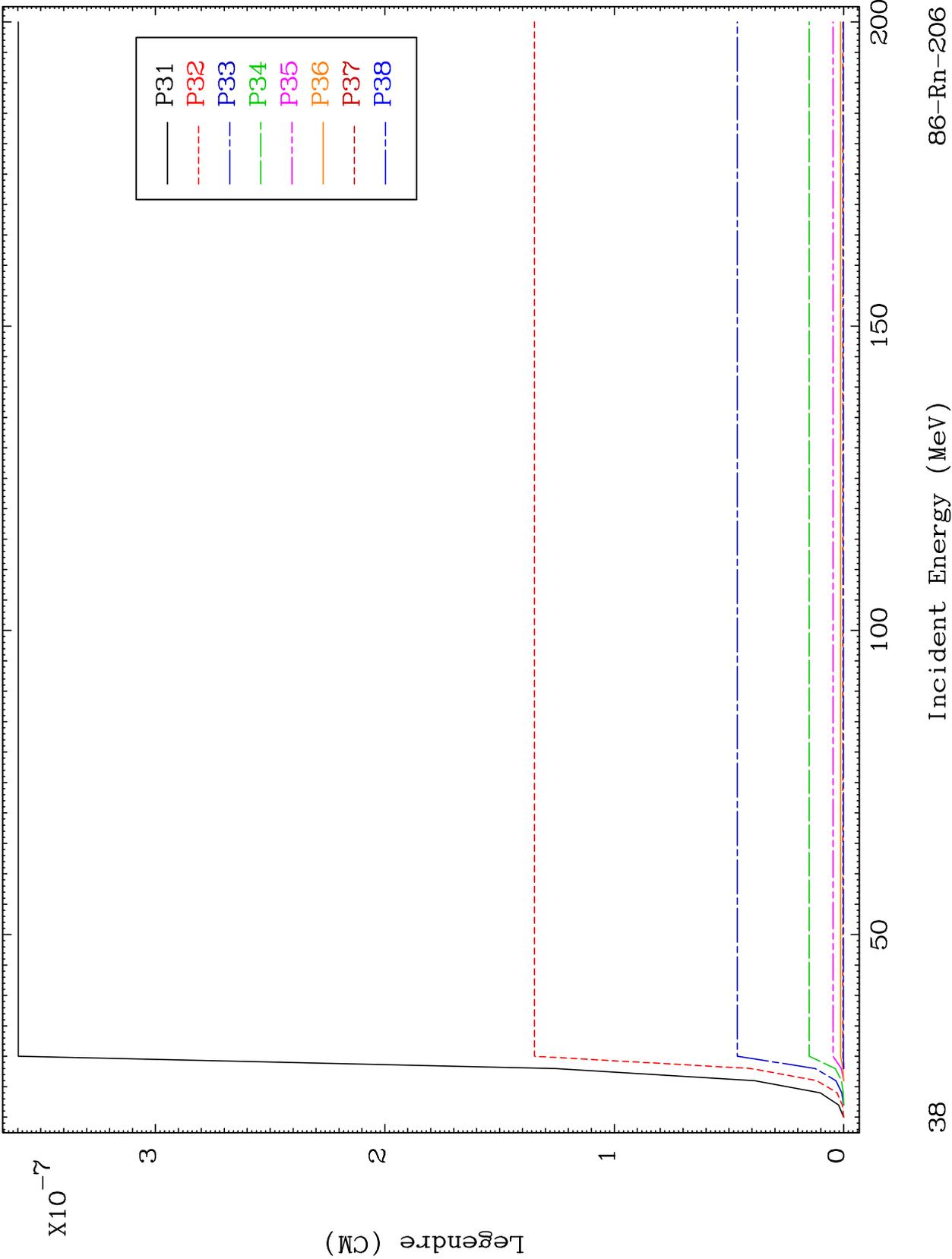




MAT 8610

MT= 55 (n,n') Level  
Legendre Coefficients

86-Rn-206



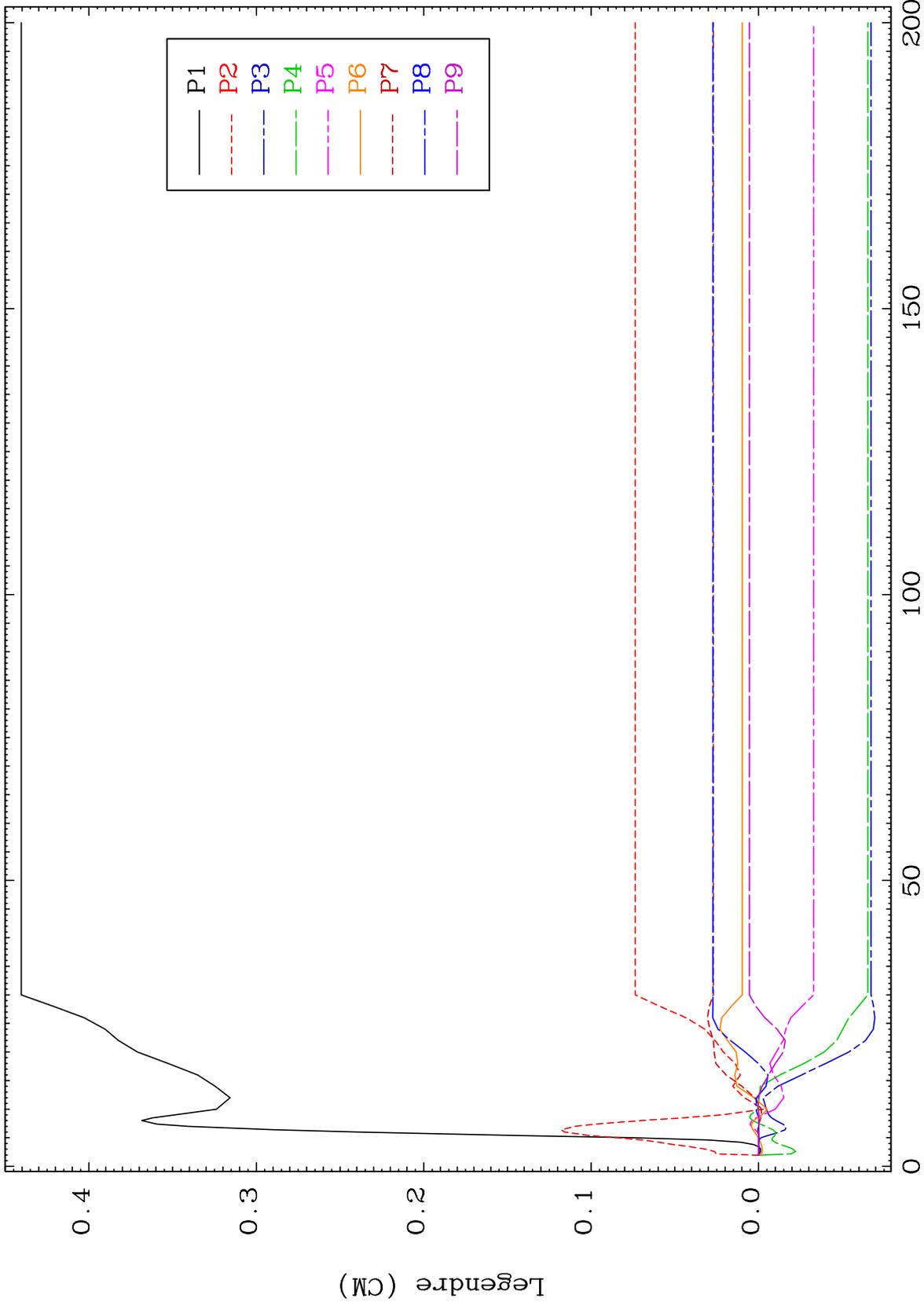
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86-Rn-206

MAT 8610

MT= 56 (n,n') Level  
Legendre Coefficients

86-Rn-206



39

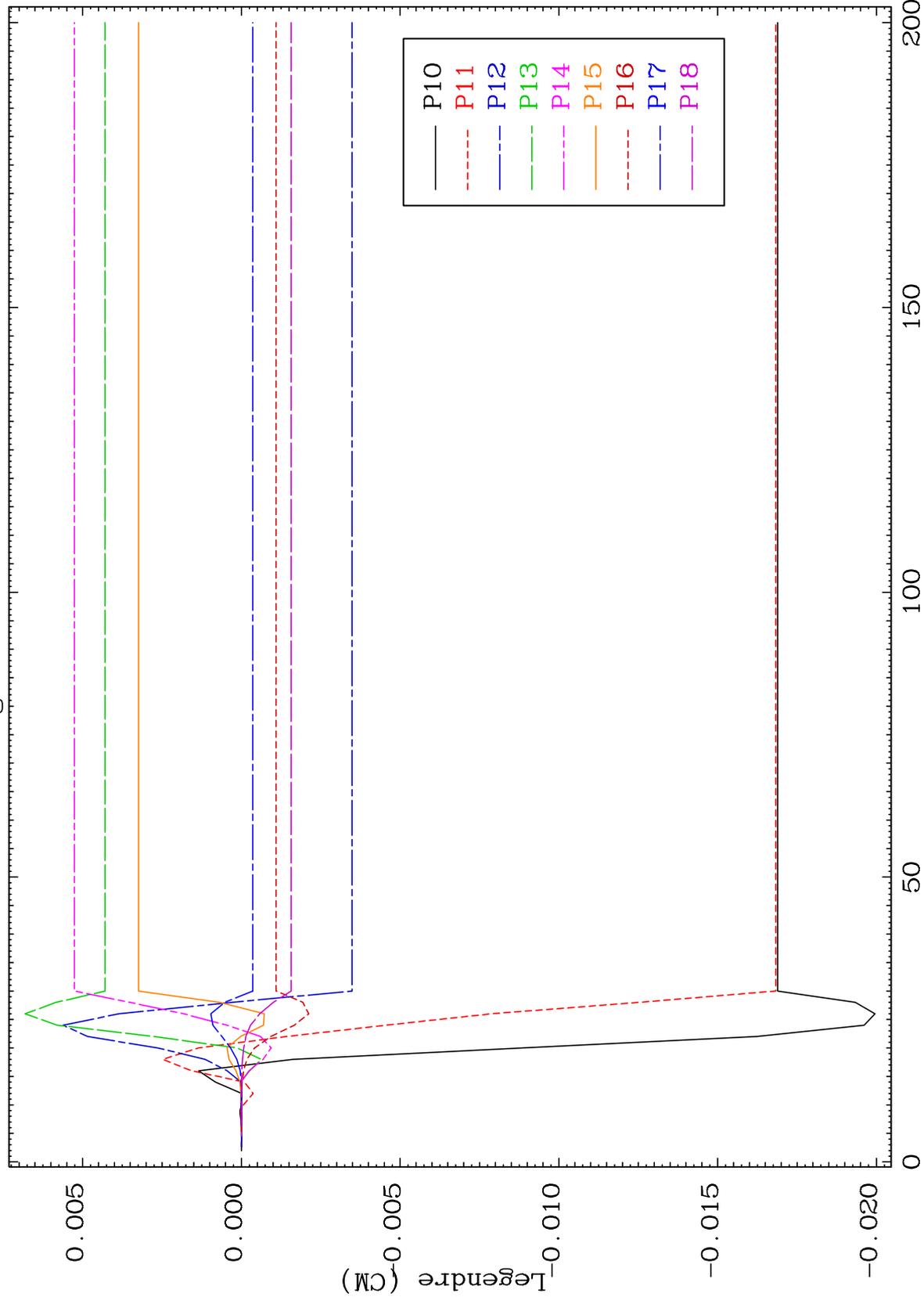
Incident Energy (MeV)

86-Rn-206

MAT 8610

MT= 56 (n,n') Level  
Legendre Coefficients

86-Rn-206



86-Rn-206

Incident Energy (MeV)

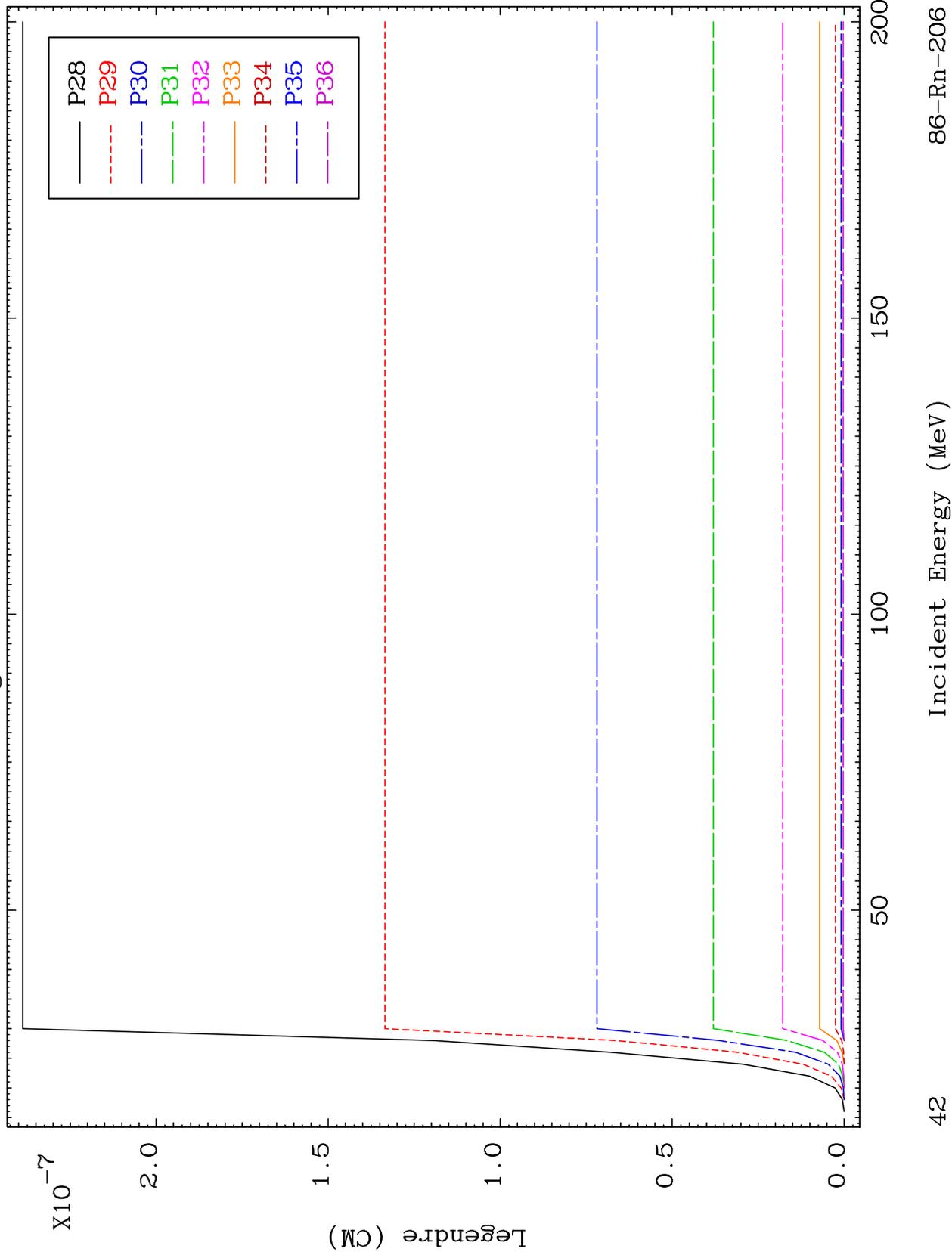
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MAT 8610

MT= 56 (n,n') Level  
Legendre Coefficients

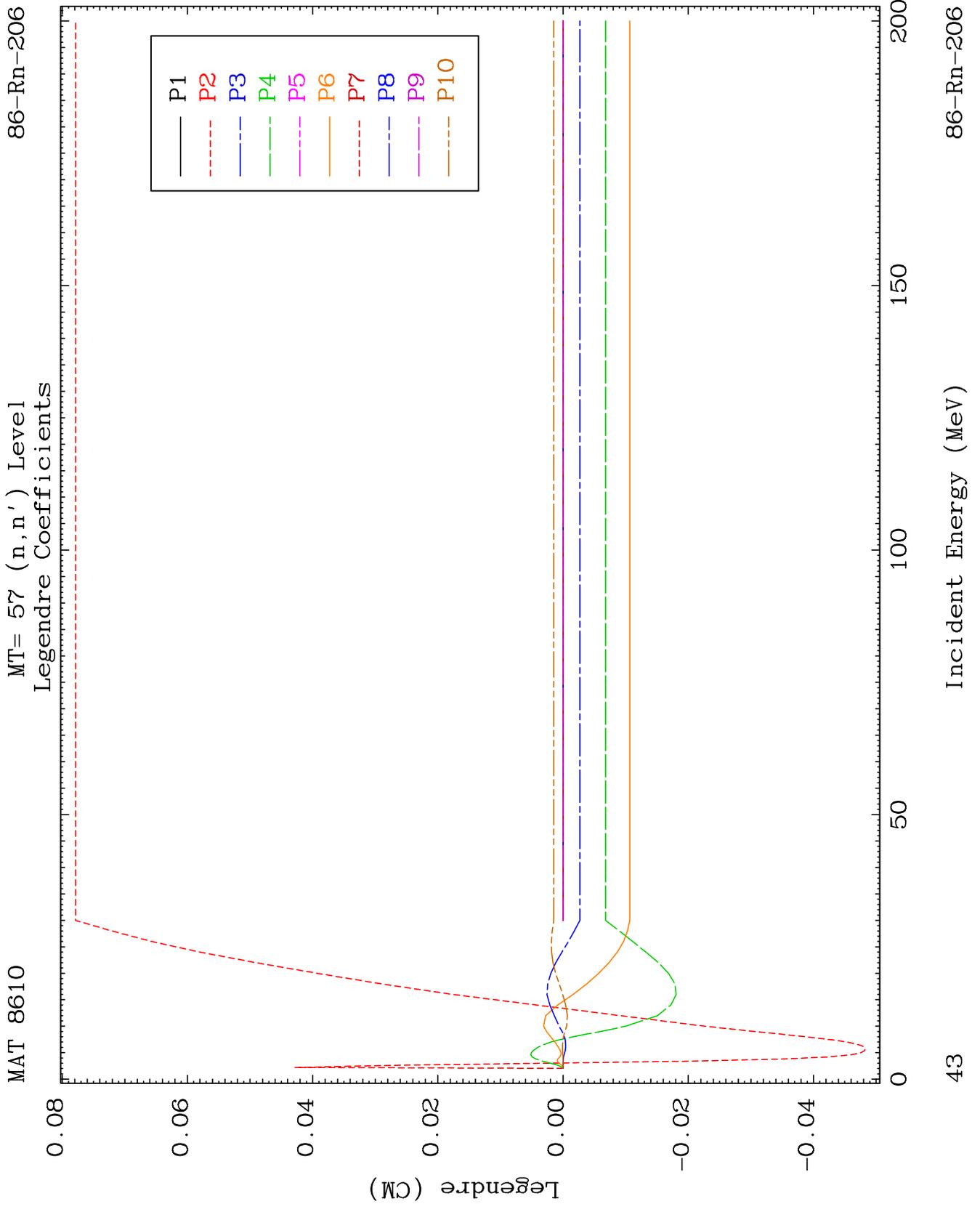
86-Rn-206



42

Incident Energy (MeV)

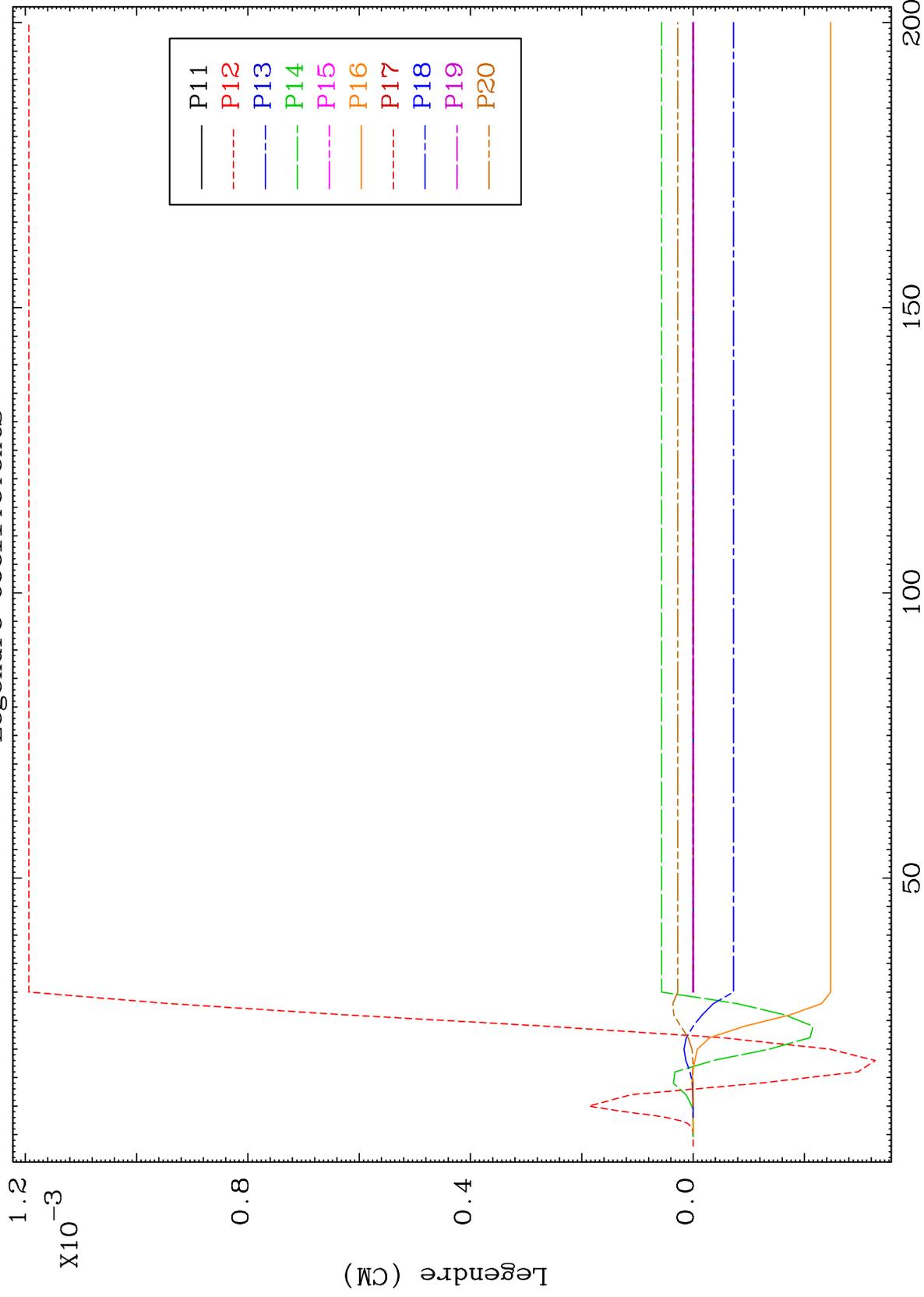
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MAT 8610

MT= 57 (n,n') Level  
Legendre Coefficients

86-Rn-206



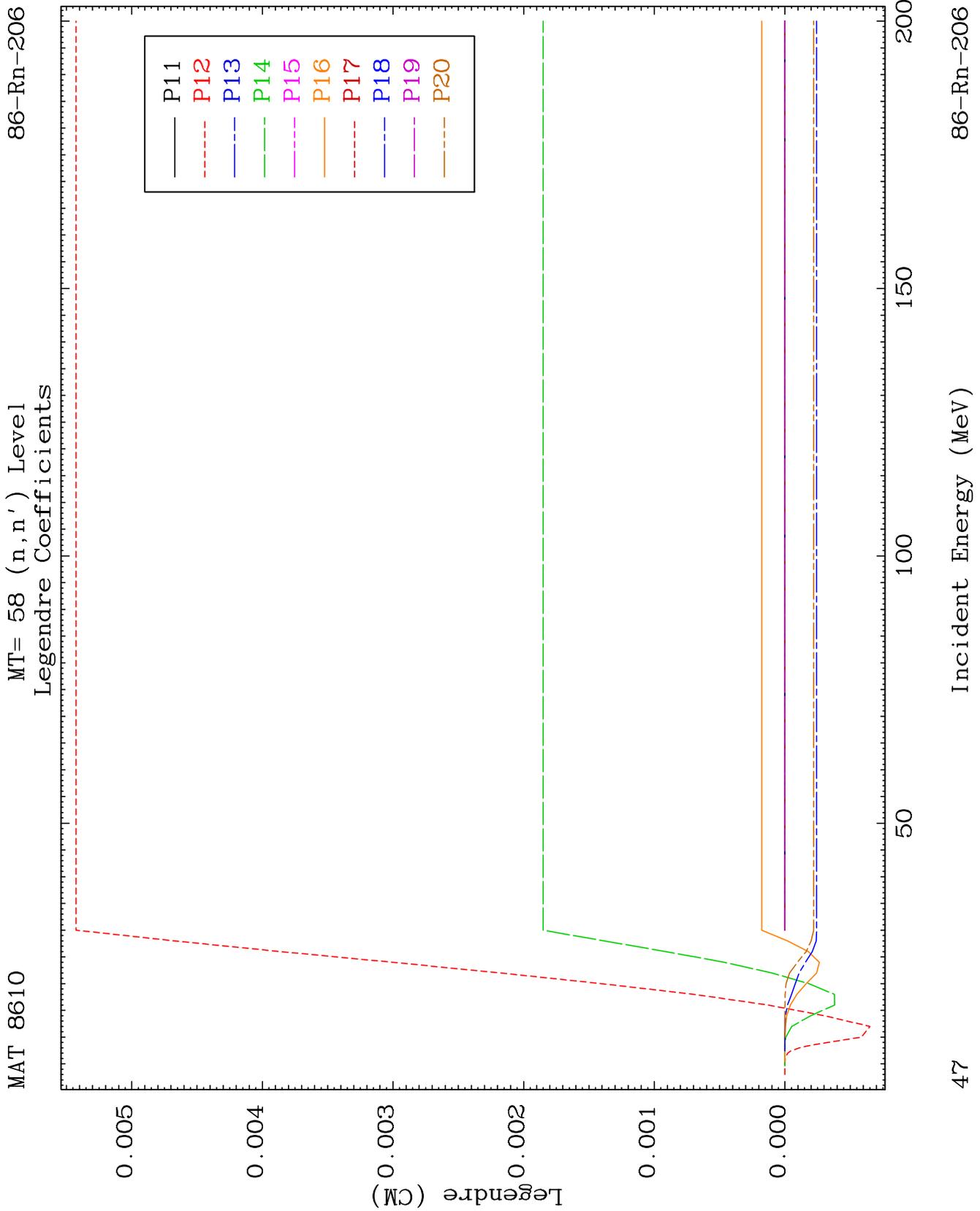
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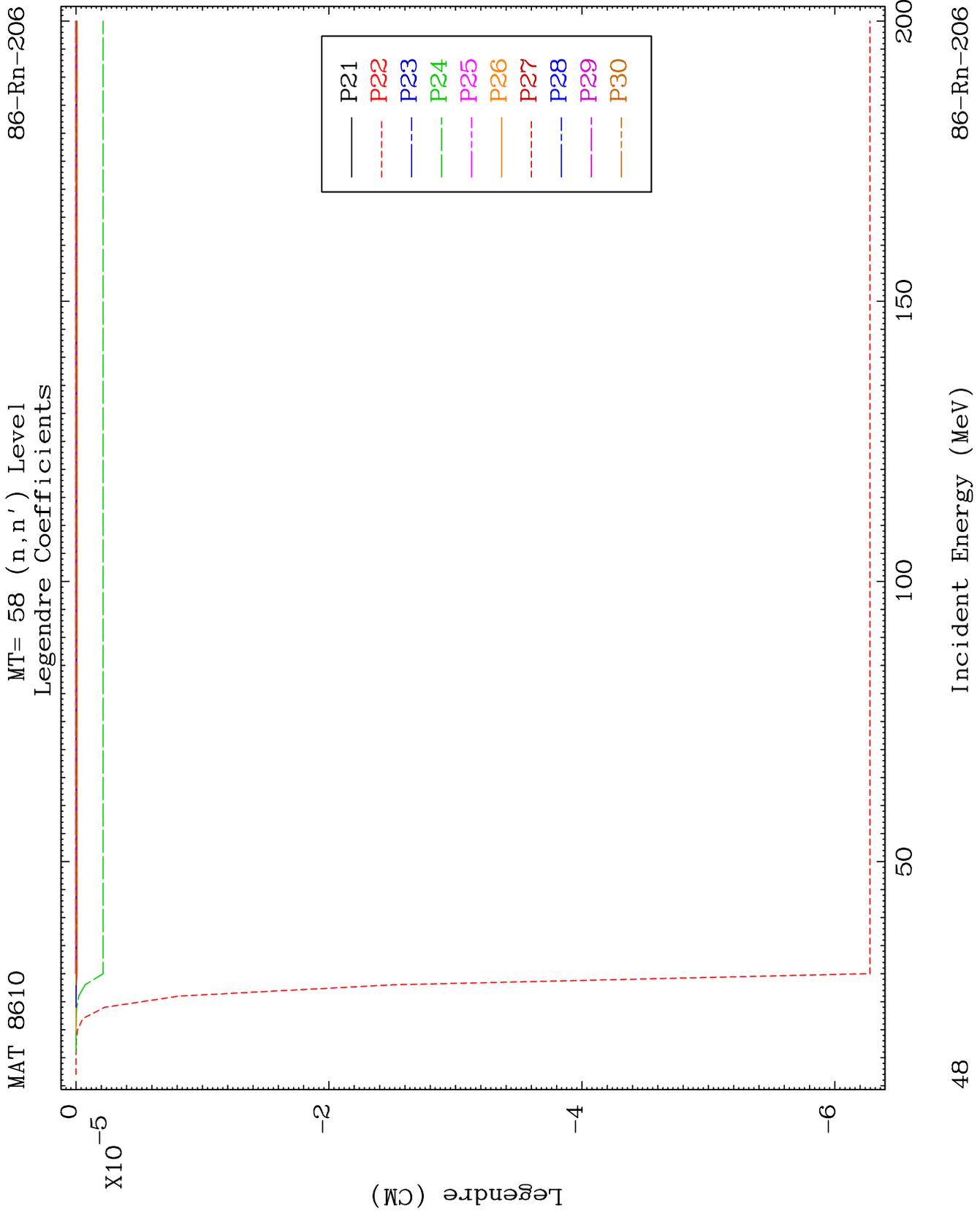
Incident Energy (MeV)

86-Rn-206





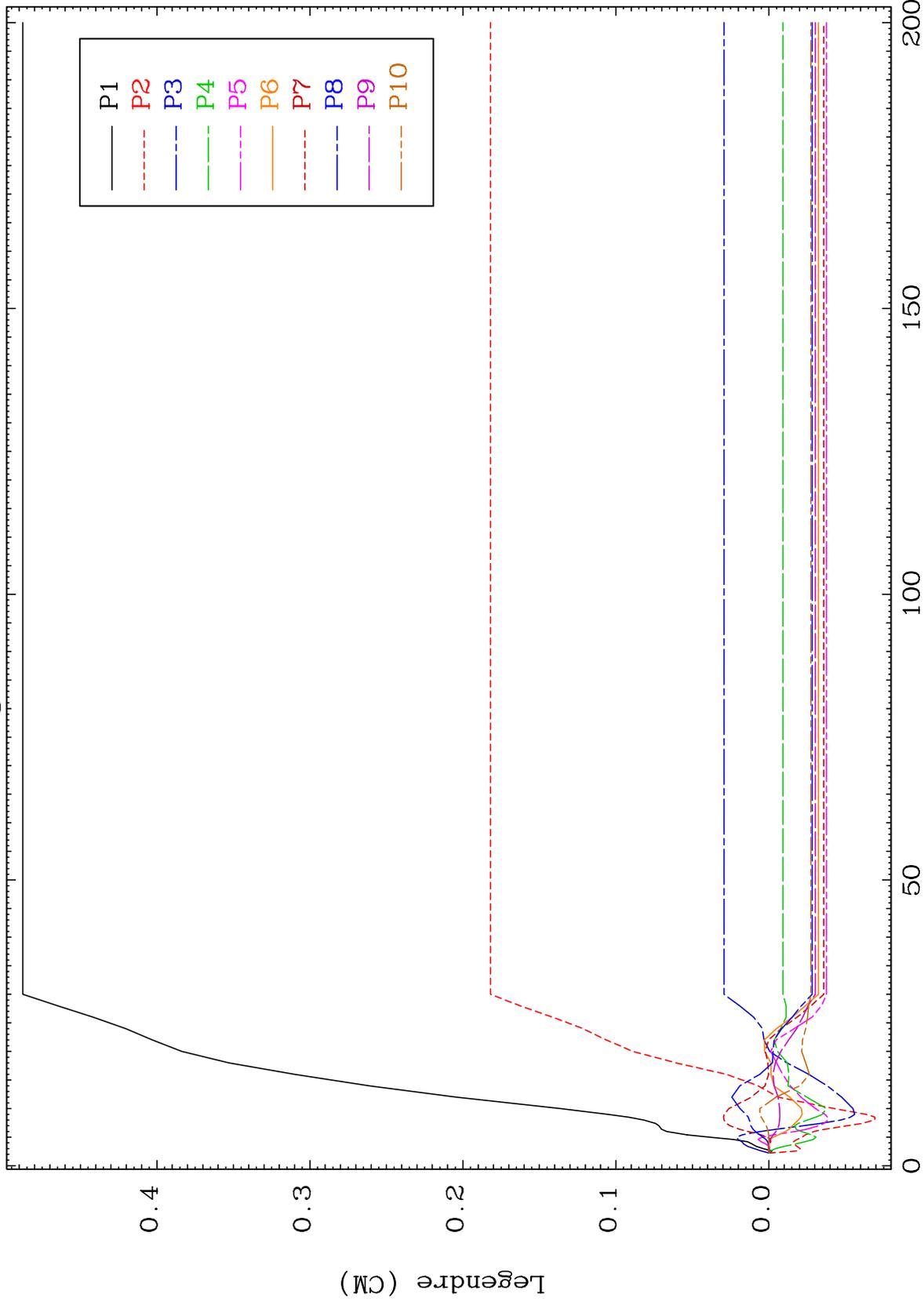




MAT 8610

MT= 59 (n,n') Level  
Legendre Coefficients

86-Rn-206



49

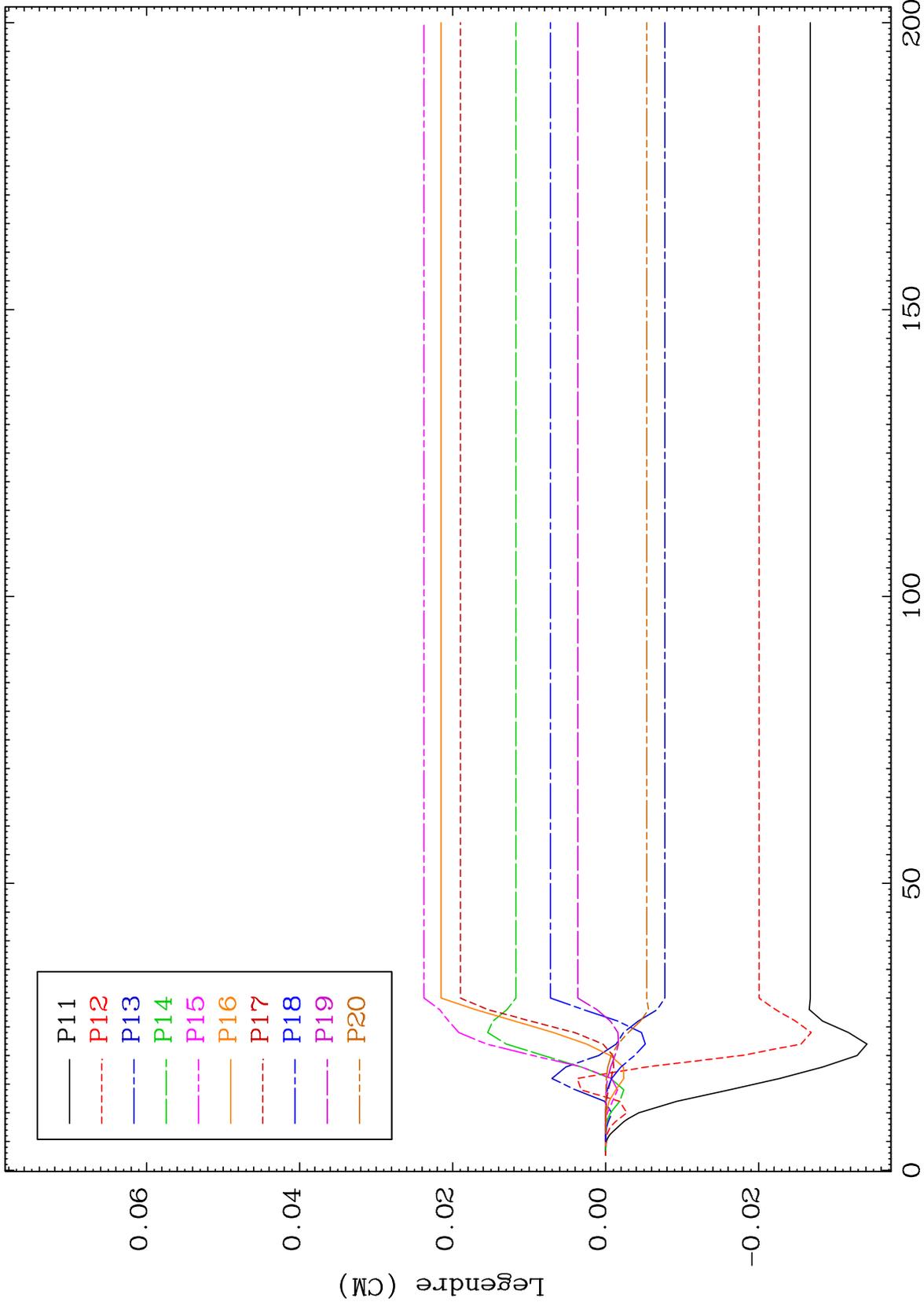
Incident Energy (MeV)

86-Rn-206

MAT 8610

MT= 59 (n,n') Level  
Legendre Coefficients

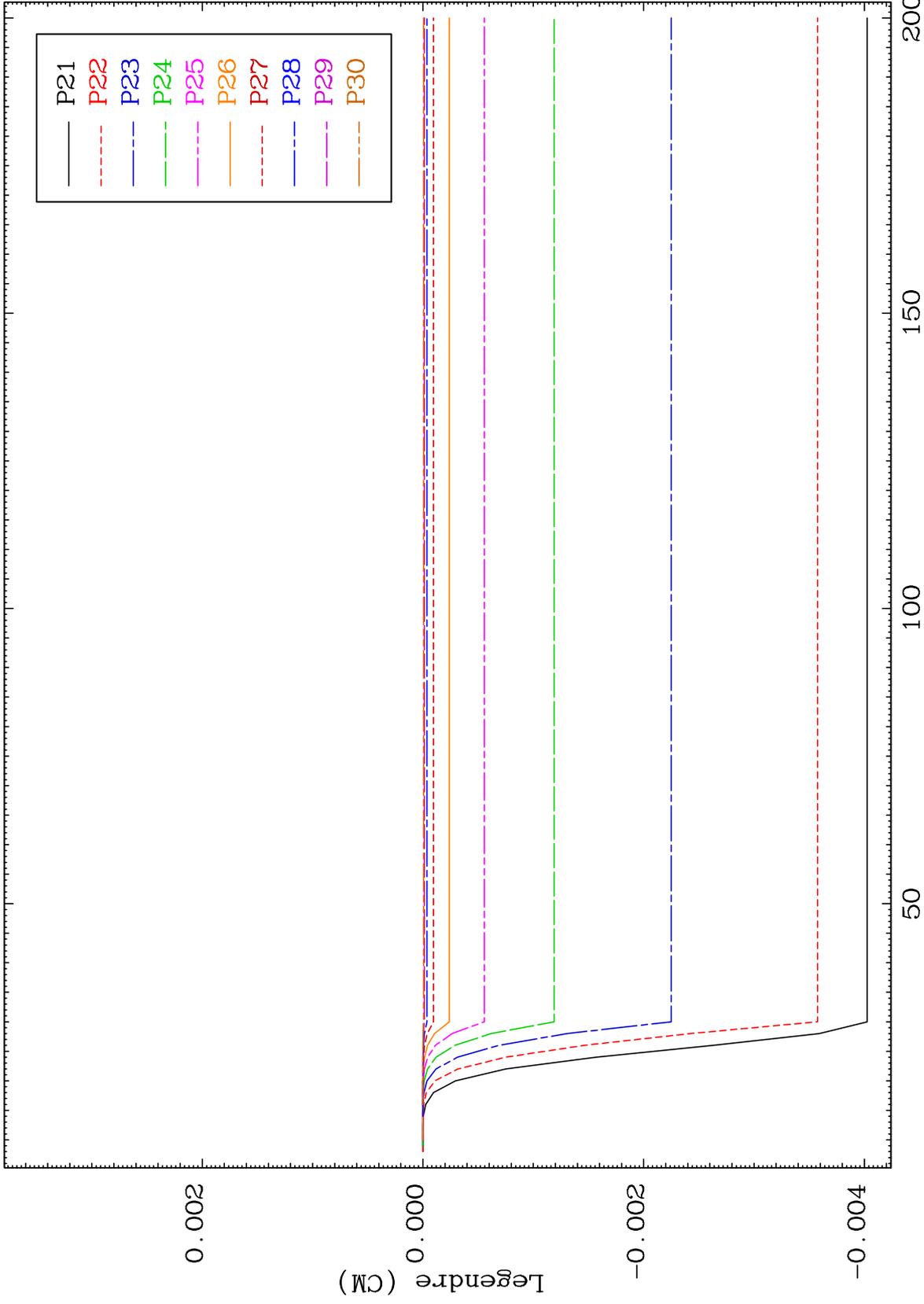
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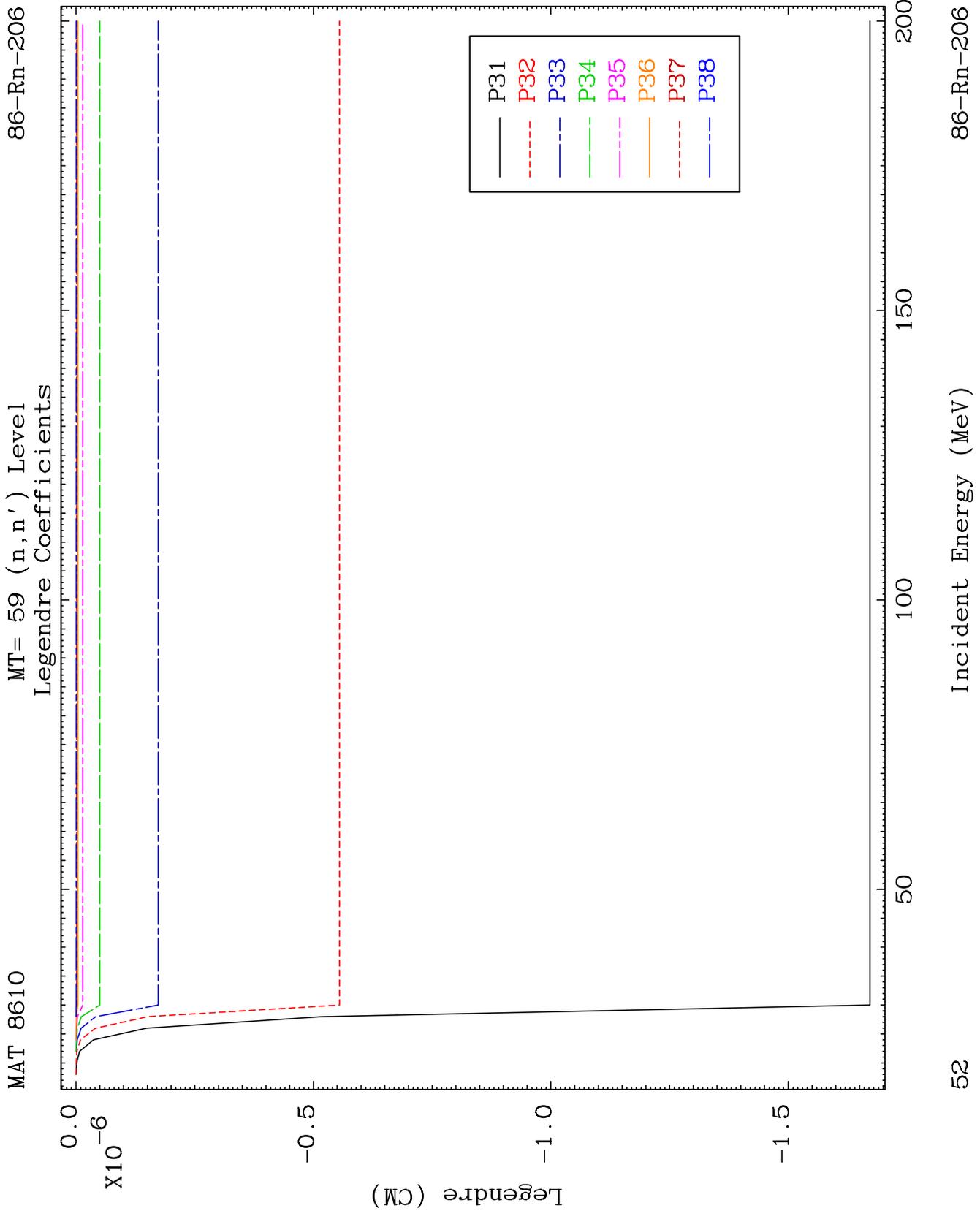


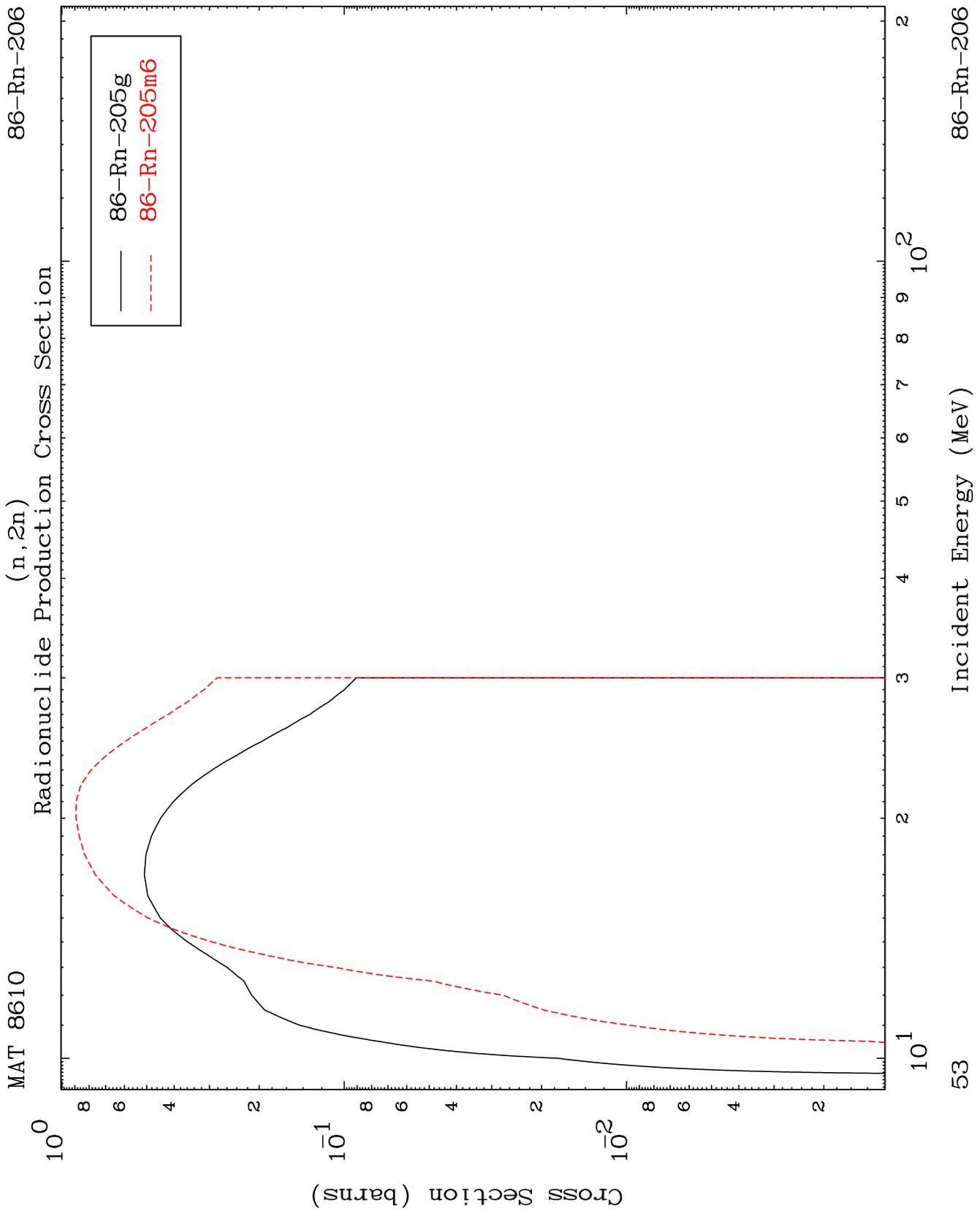
86-Rn-206

Incident Energy (MeV)

50





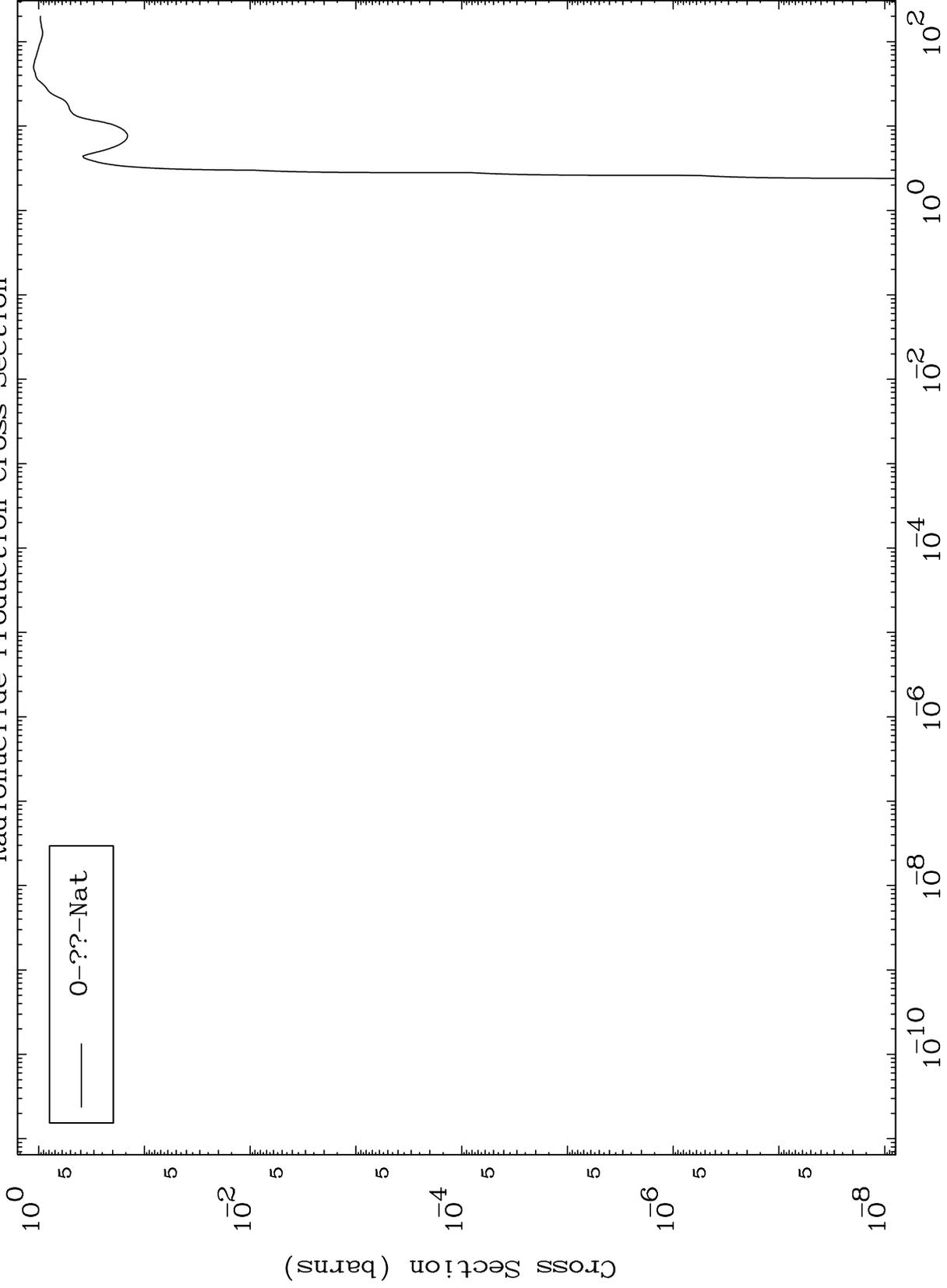


MAT 8610

Fission

86-Rn-206

Radionuclide Production Cross Section

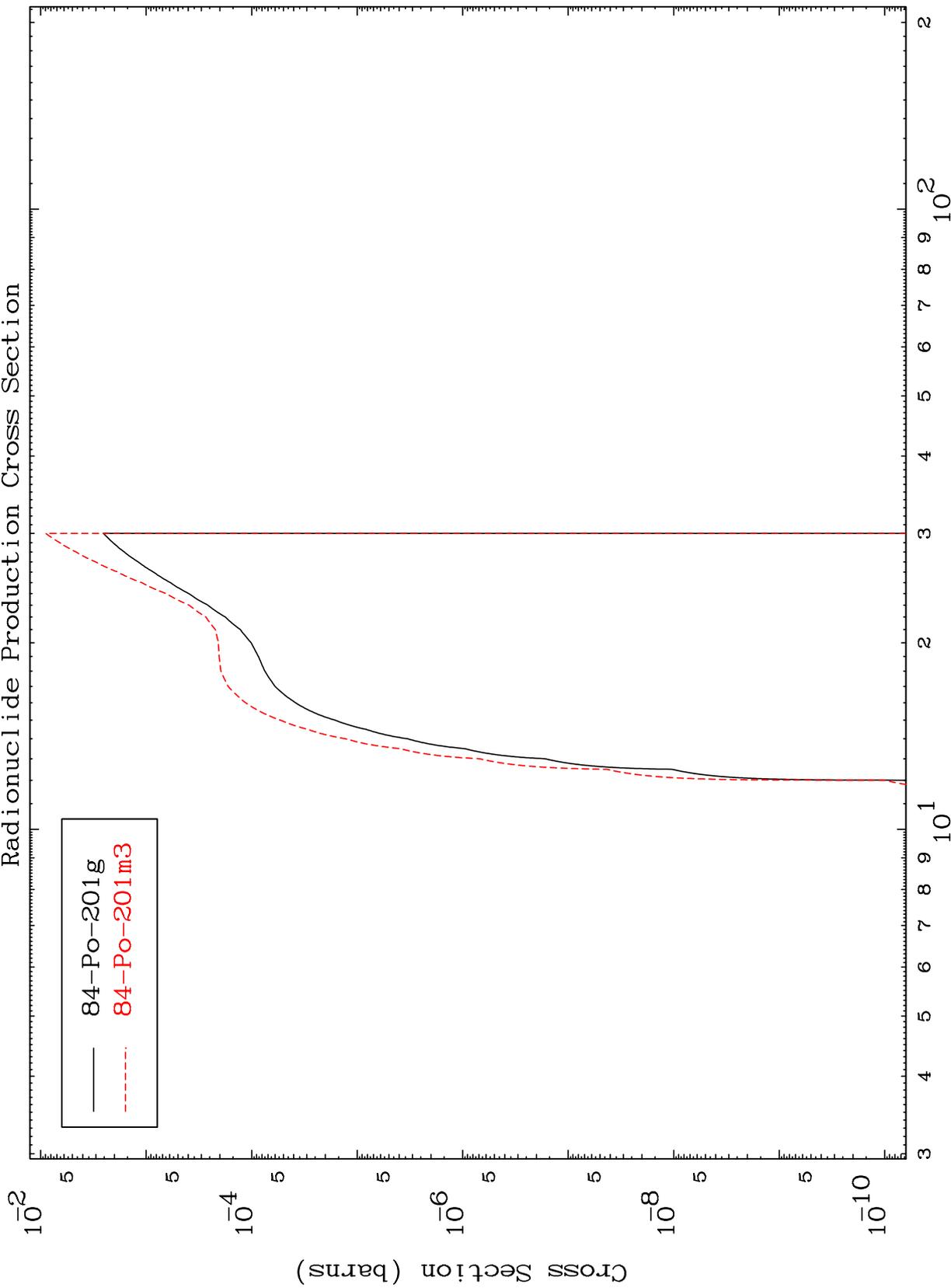


MAT 8610

$(n, 2n) \alpha$

86-Rn-206

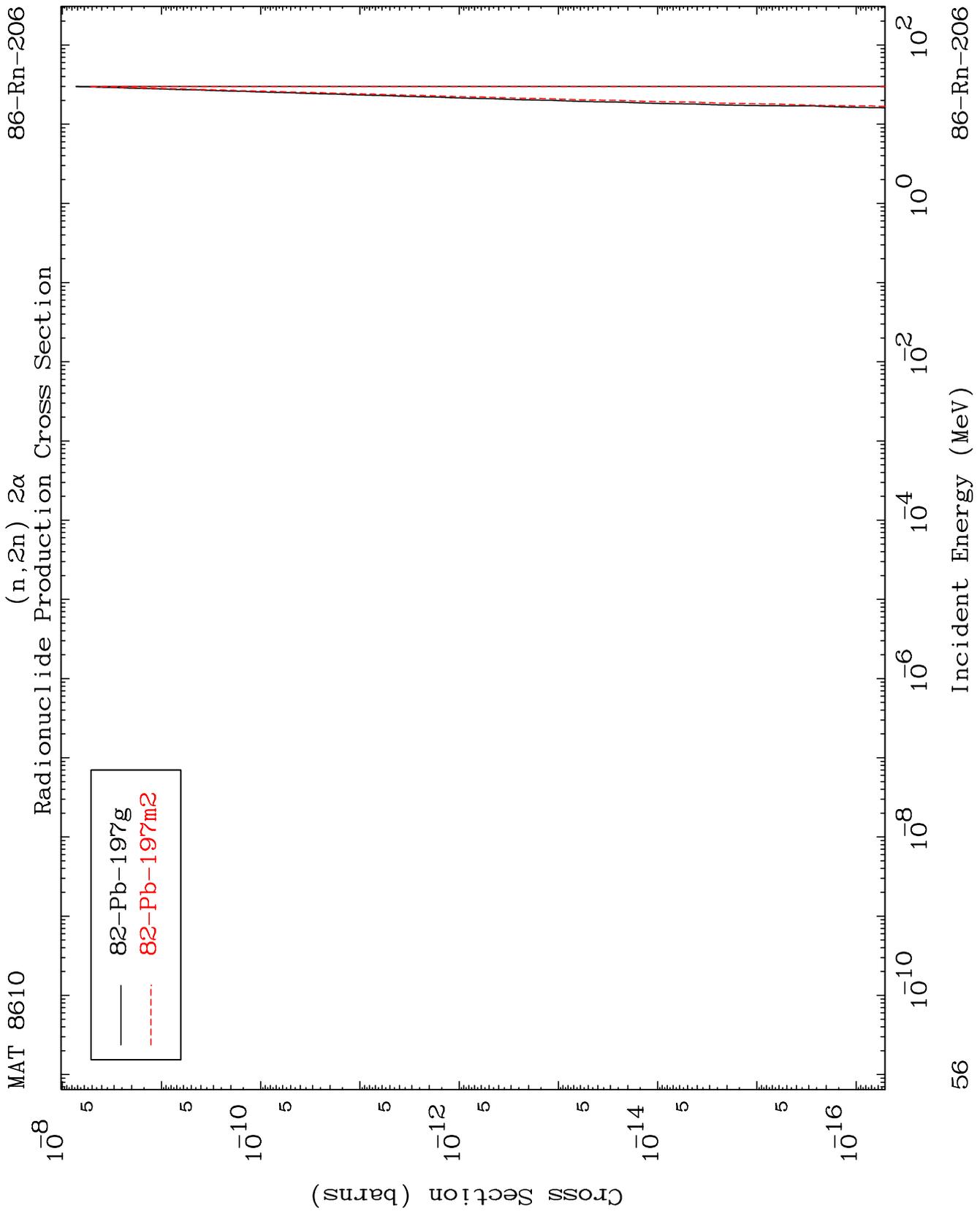
Radionuclide Production Cross Section



Incident Energy (MeV)

86-Rn-206

55

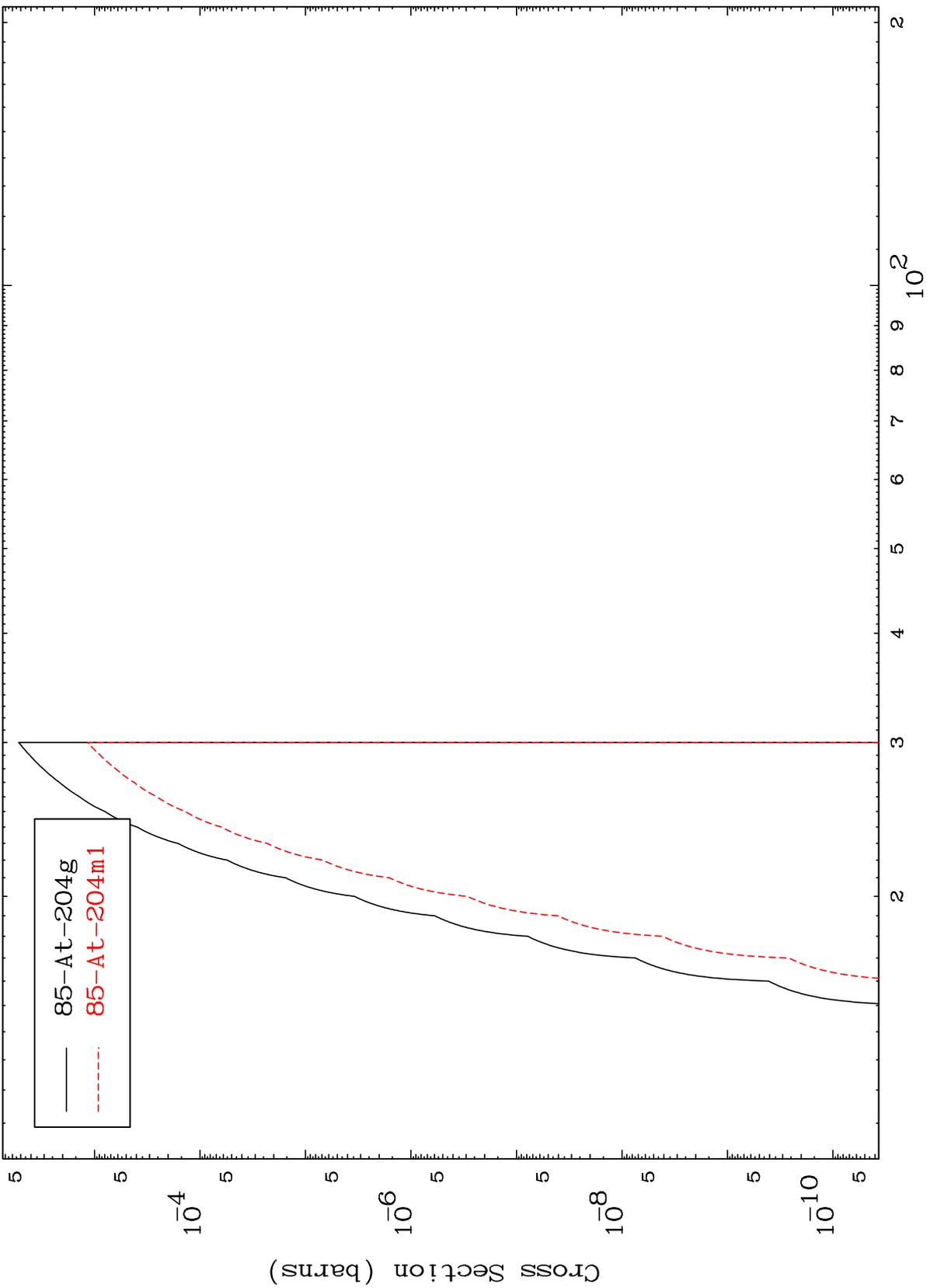


MAT 8610

(n,n') d

86-Rn-206

Radionuclide Production Cross Section



57

Incident Energy (MeV)

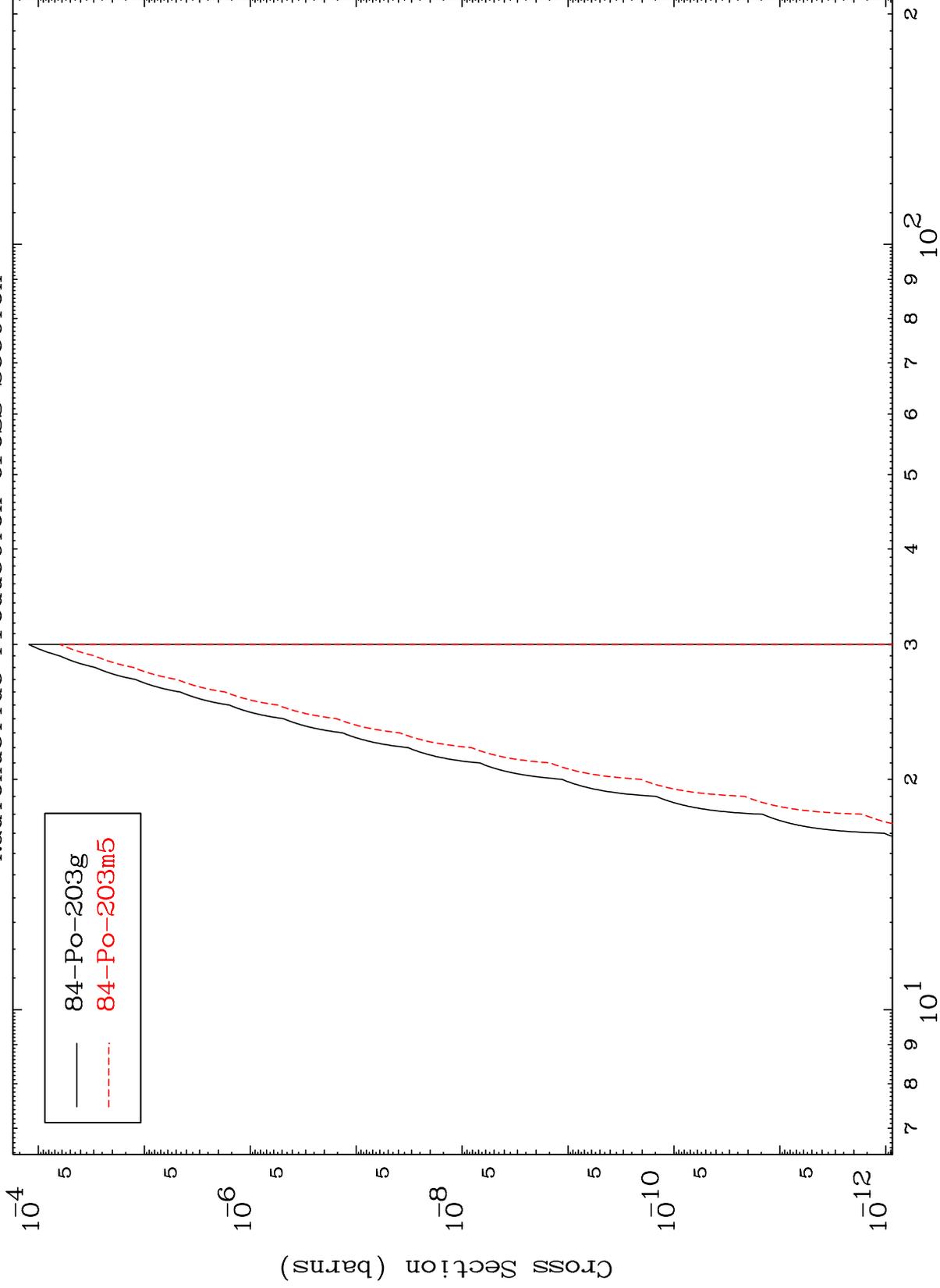
86-Rn-206

MAT 8610

(n,n') He-3

86-Rn-206

Radionuclide Production Cross Section



58

Incident Energy (MeV)

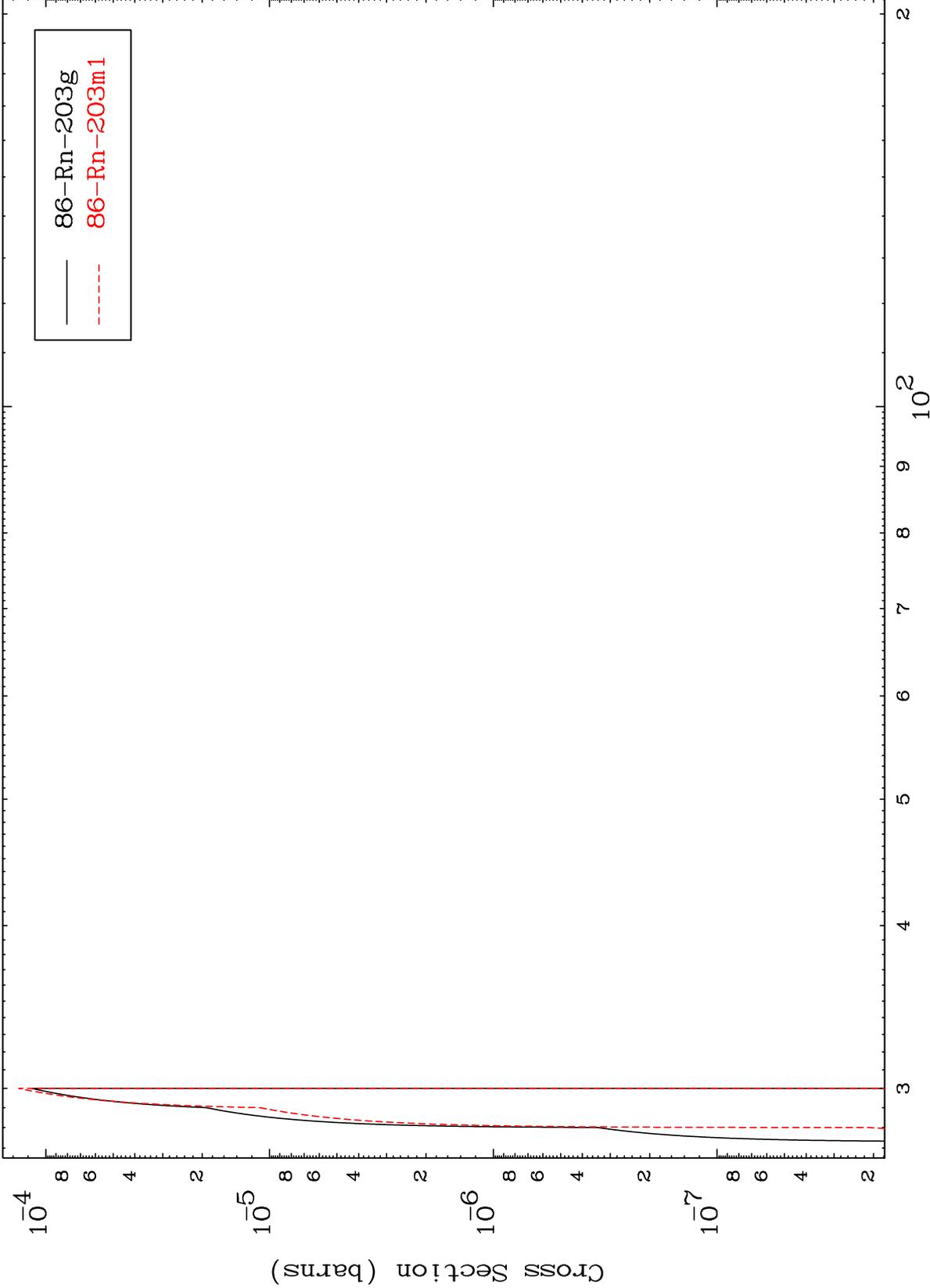
86-Rn-206

MAT 8610

(n,4n)

86-Rn-206

Radionuclide Production Cross Section



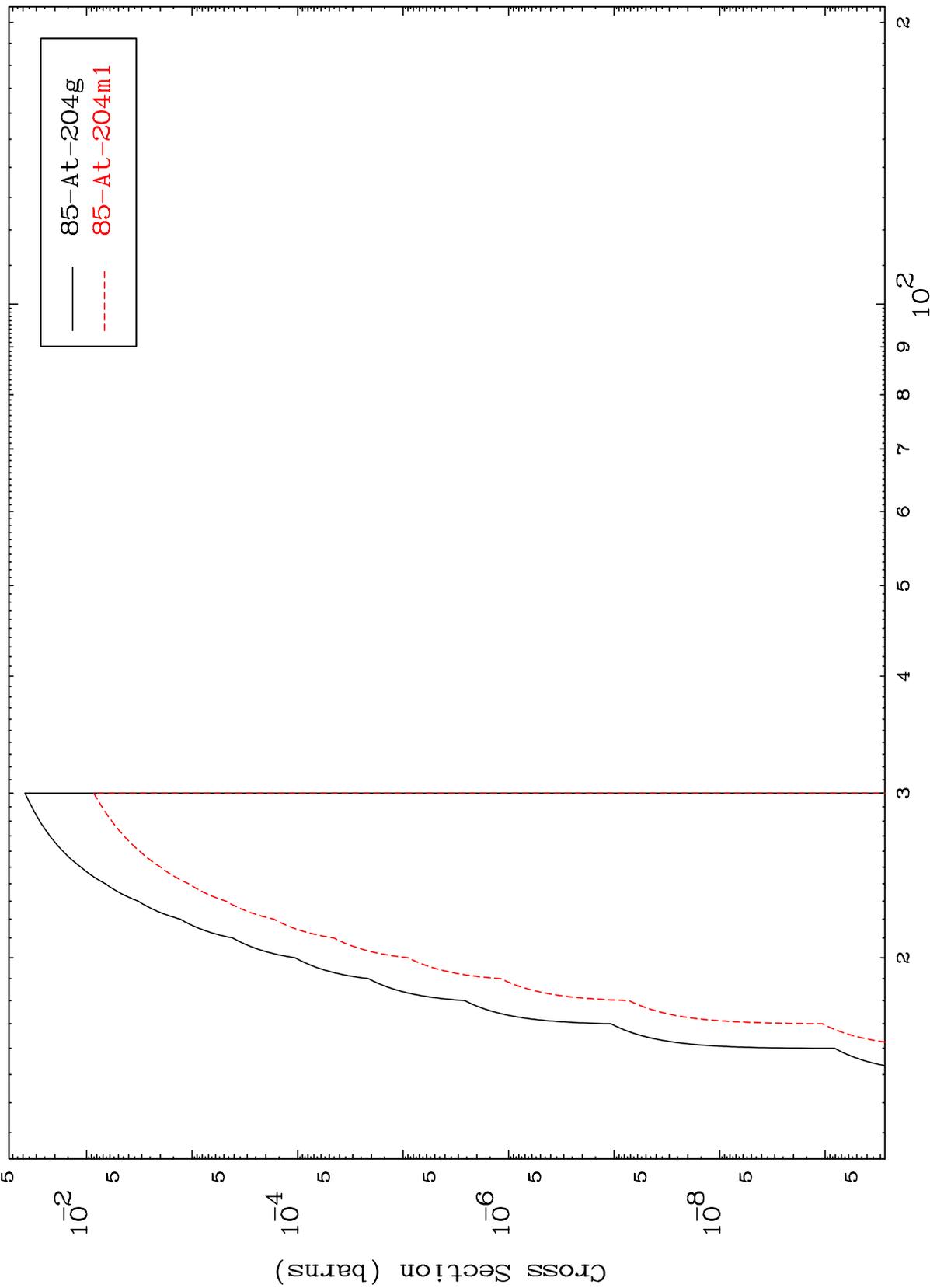
86-Rn-203g  
86-Rn-203m1

MAT 8610

(n,2n) p

86-Rn-206

Radionuclide Production Cross Section



60

Incident Energy (MeV)

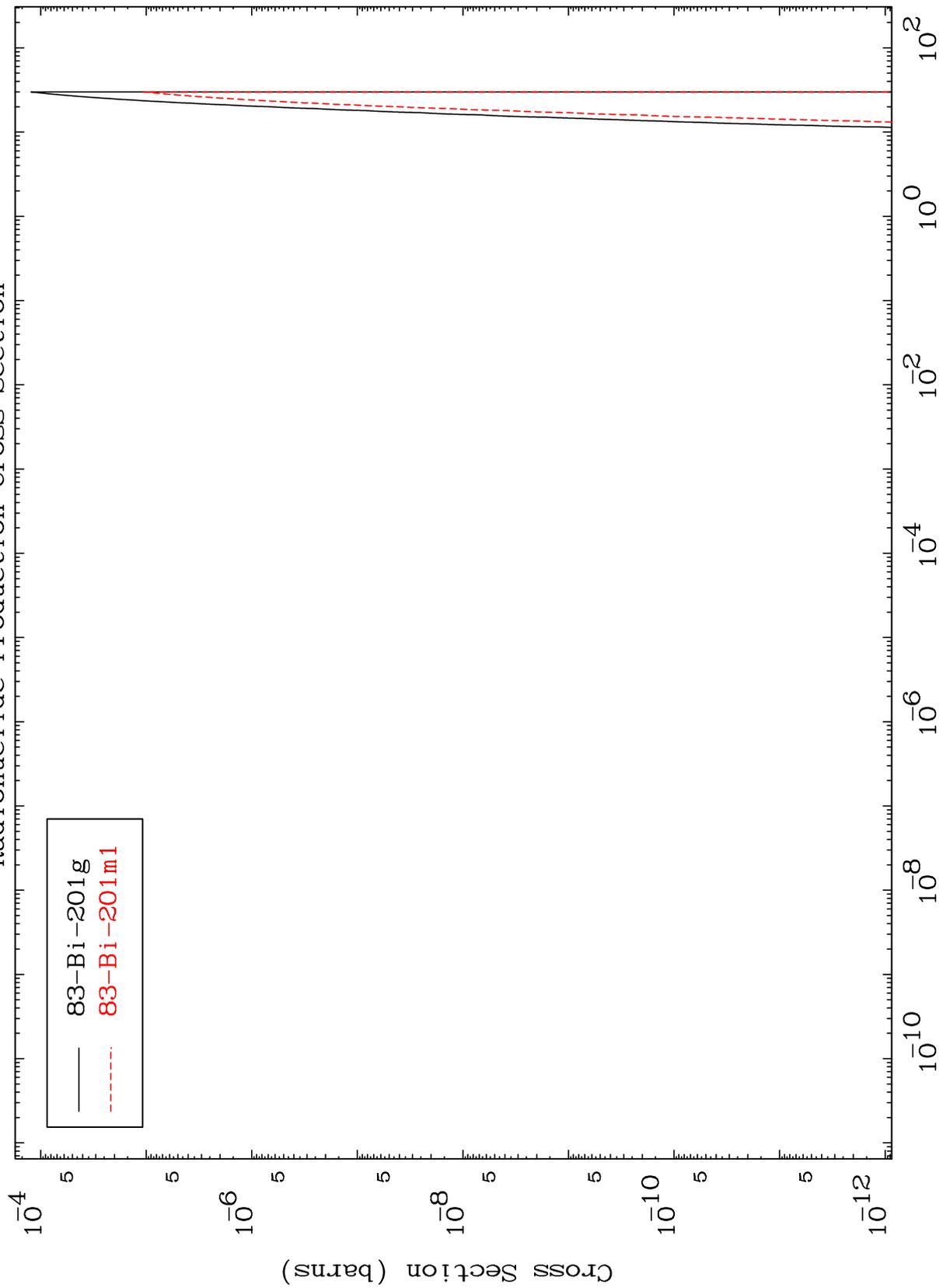
86-Rn-206

MAT 8610

(n,n') p  $\alpha$

86-Rn-206

Radionuclide Production Cross Section



— 83-Bi-201g  
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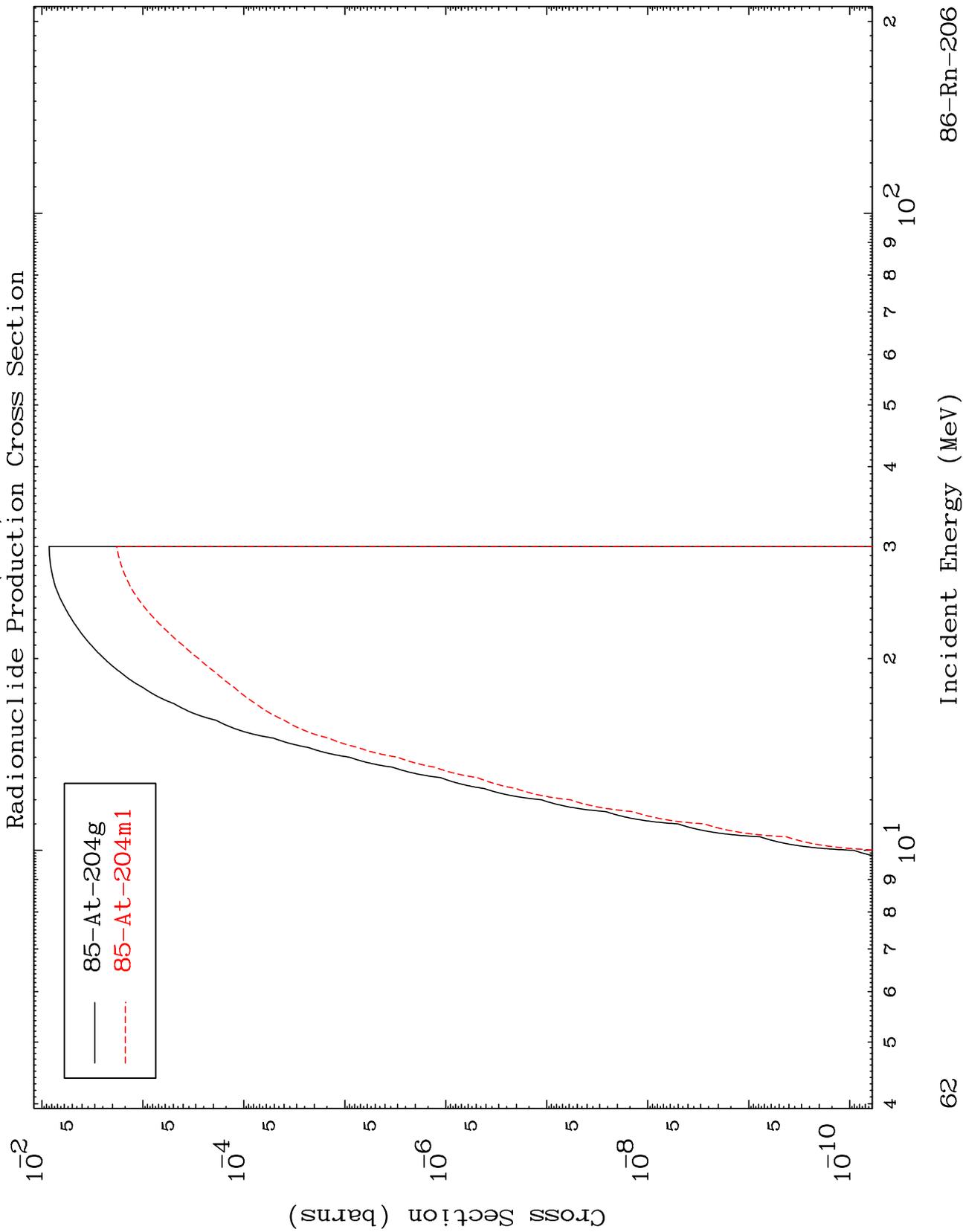
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Incident Energy (MeV)

86-Rn-206

MAT 8610

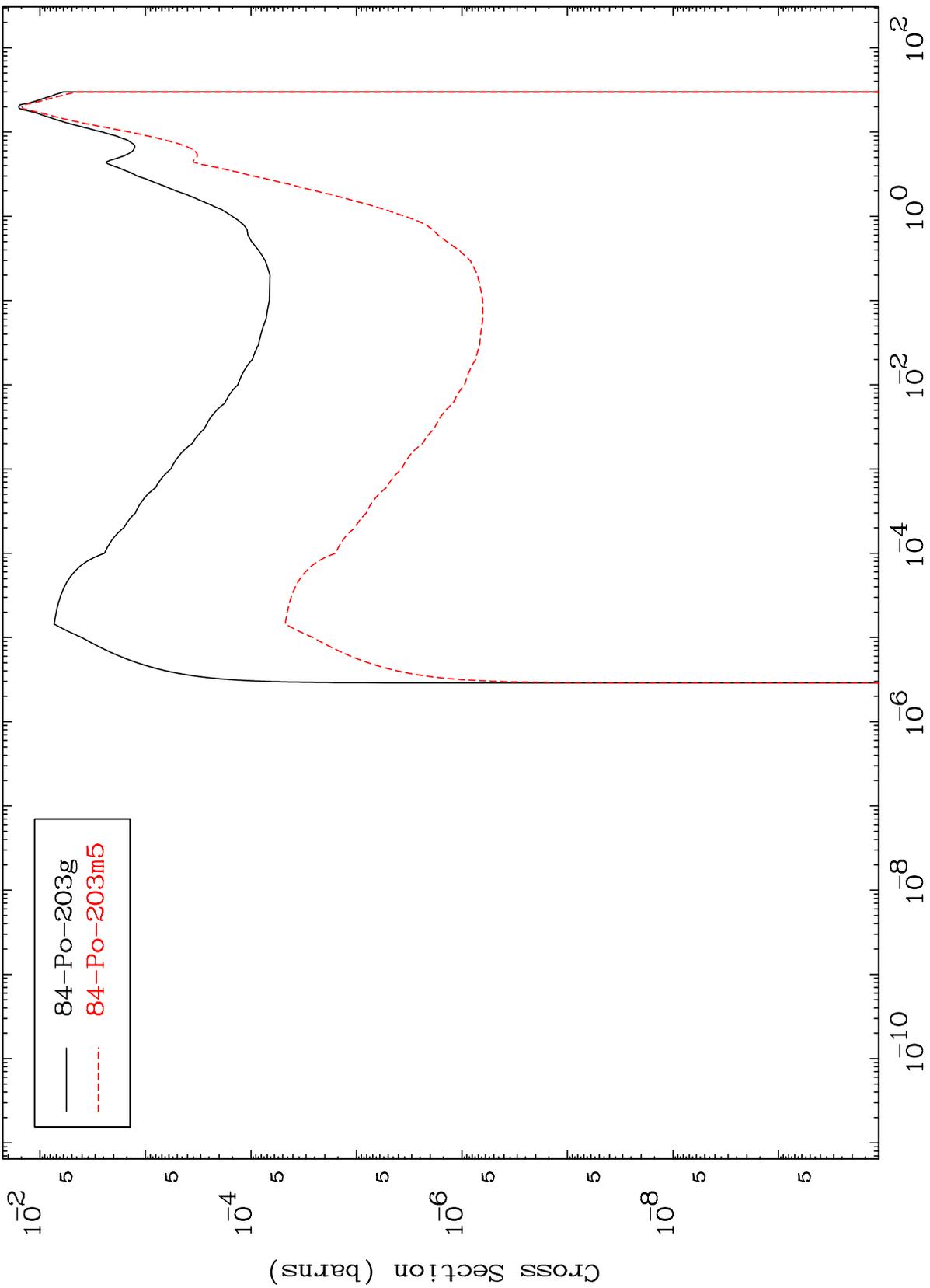
86-Rn-206



MAT 8610

86-Rn-206

(n,  $\alpha$ )  
Radionuclide Production Cross Section



— 84-Po-203g  
- - - 84-Po-203m5

63

86-Rn-206

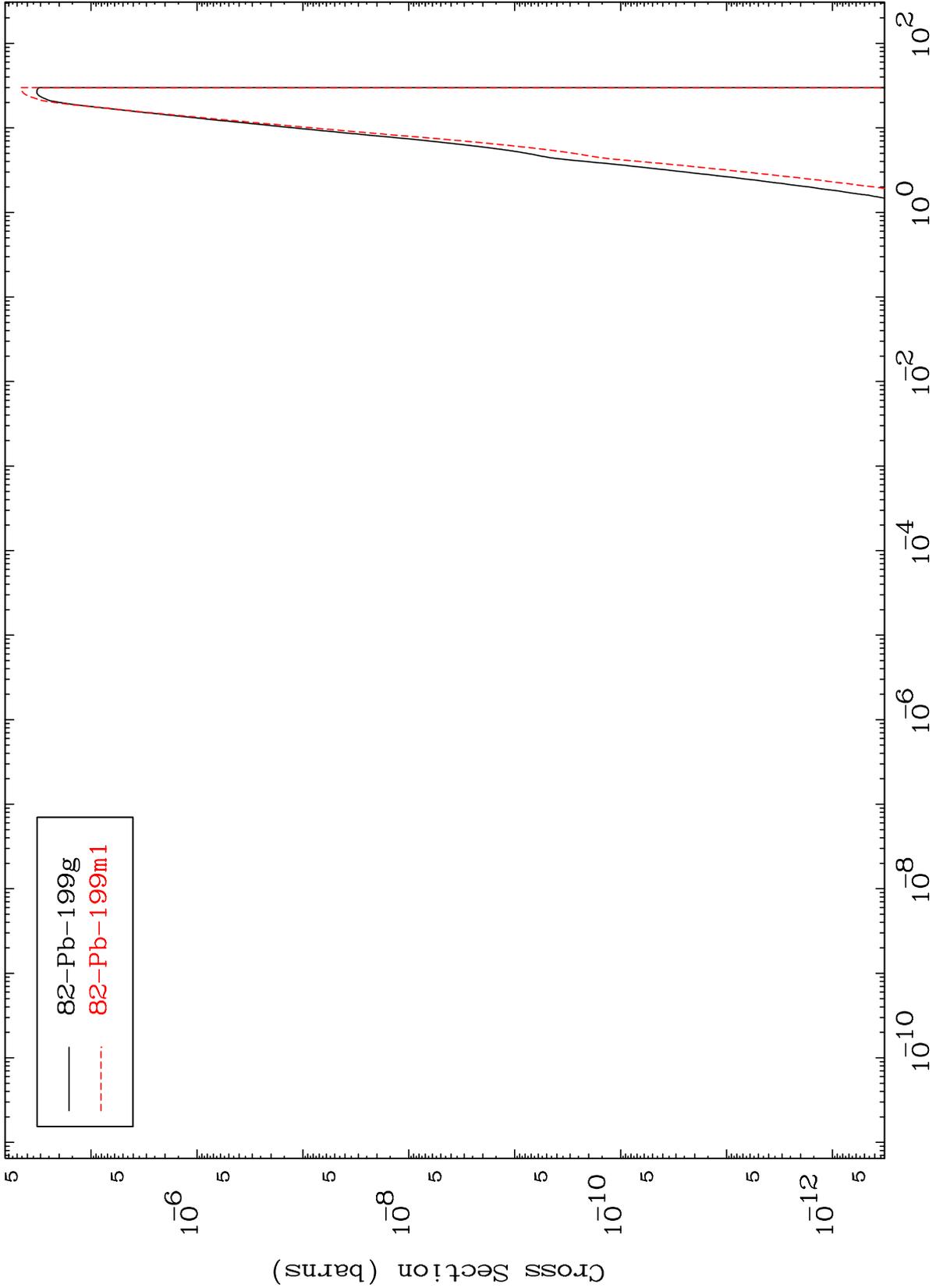
Incident Energy (MeV)

MAT 8610

(n,2α)

86-Rn-206

Radionuclide Production Cross Section



64

Incident Energy (MeV)

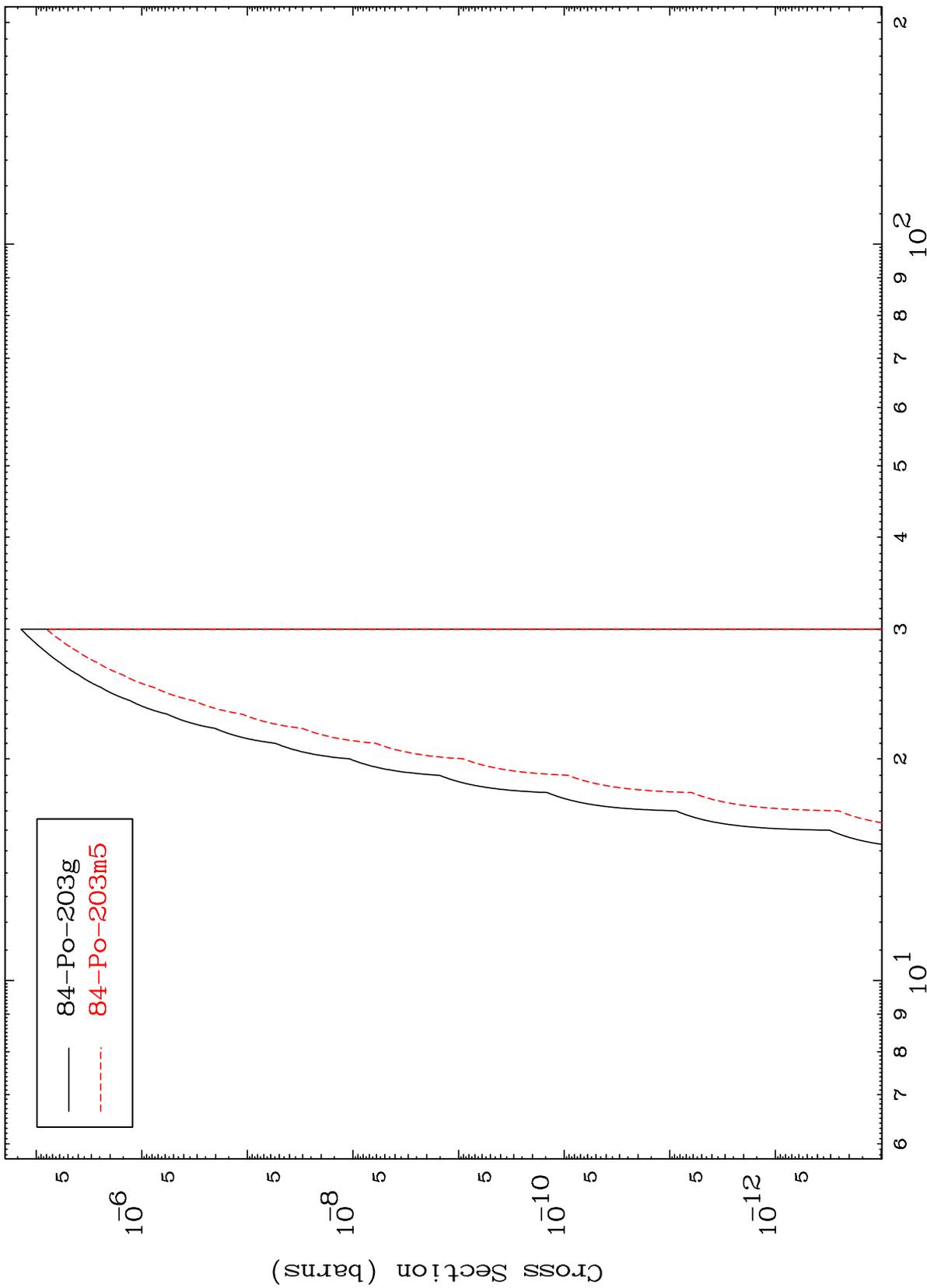
86-Rn-206

MAT 8610

(n,p) t

86-Rn-206

Radionuclide Production Cross Section



65

Incident Energy (MeV)

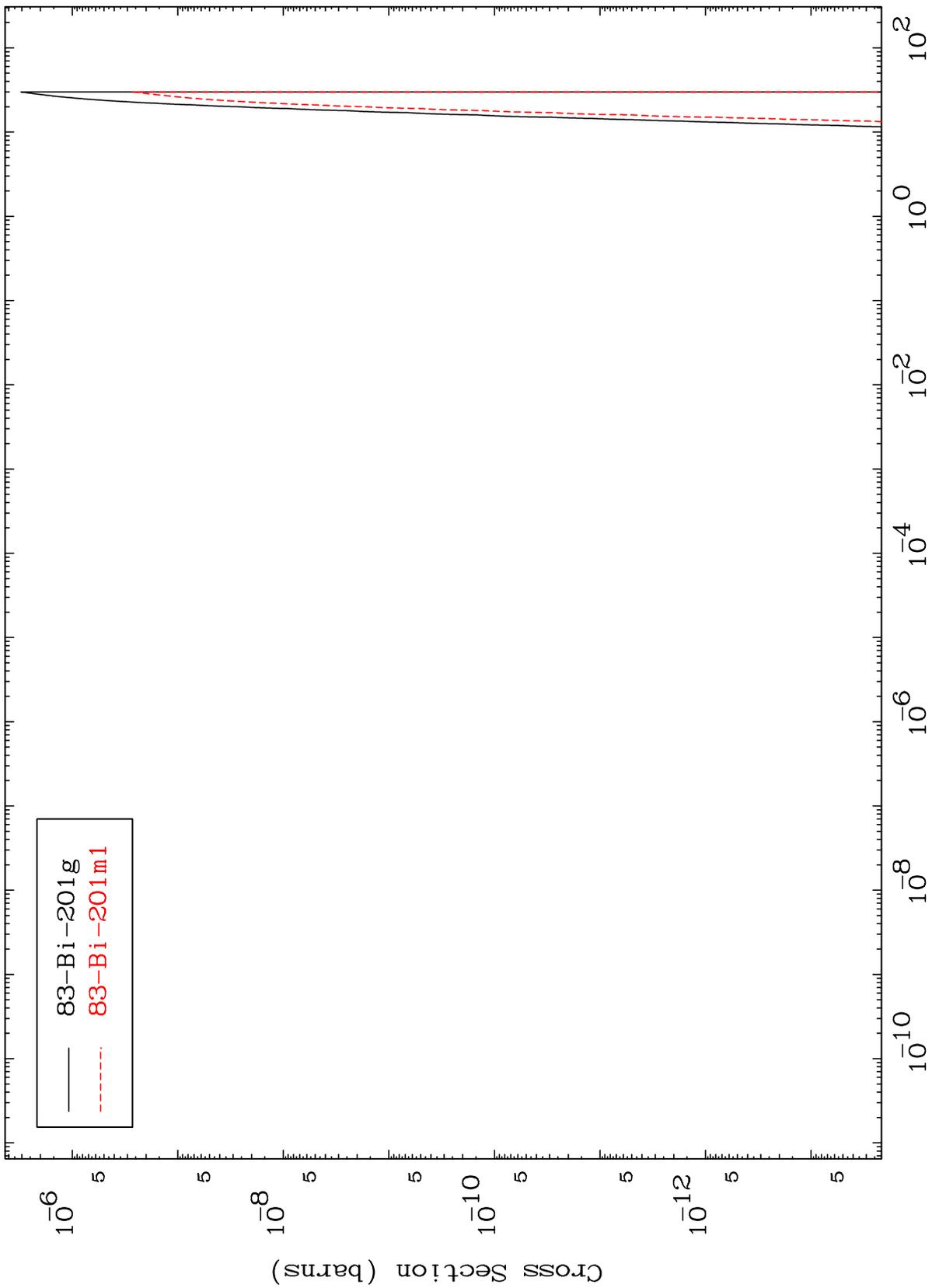
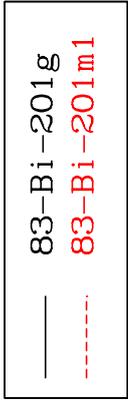
86-Rn-206

MAT 8610

(n,d)  $\alpha$

86-Rn-206

Radionuclide Production Cross Section



66

Incident Energy (MeV)

86-Rn-206