

Program EVALPLOT
(Version 2021-1)

by

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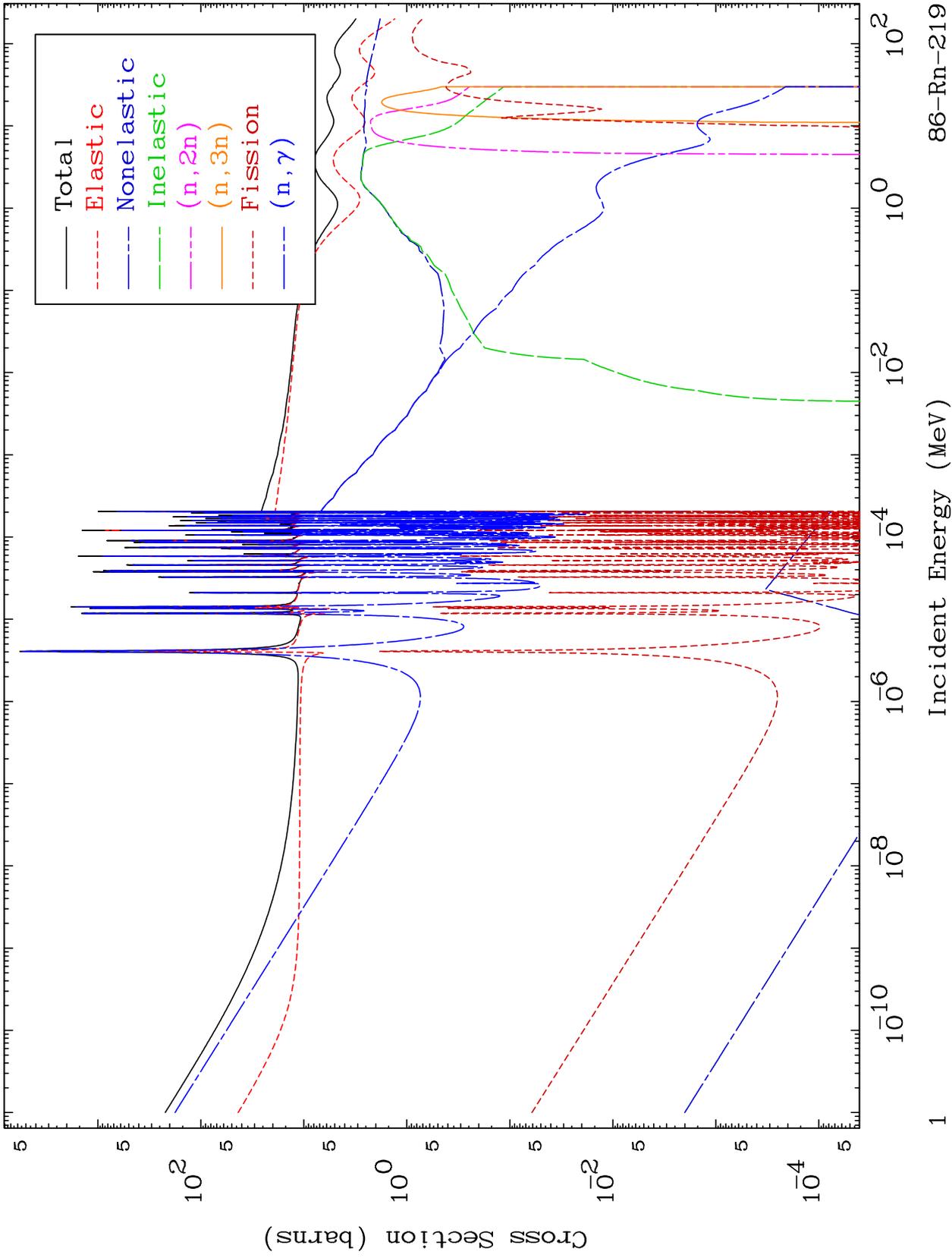
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Press Mouse Button to Start

MAT 8649

Neutron Major
293 Kelvin Cross Sections

86-Rn-219

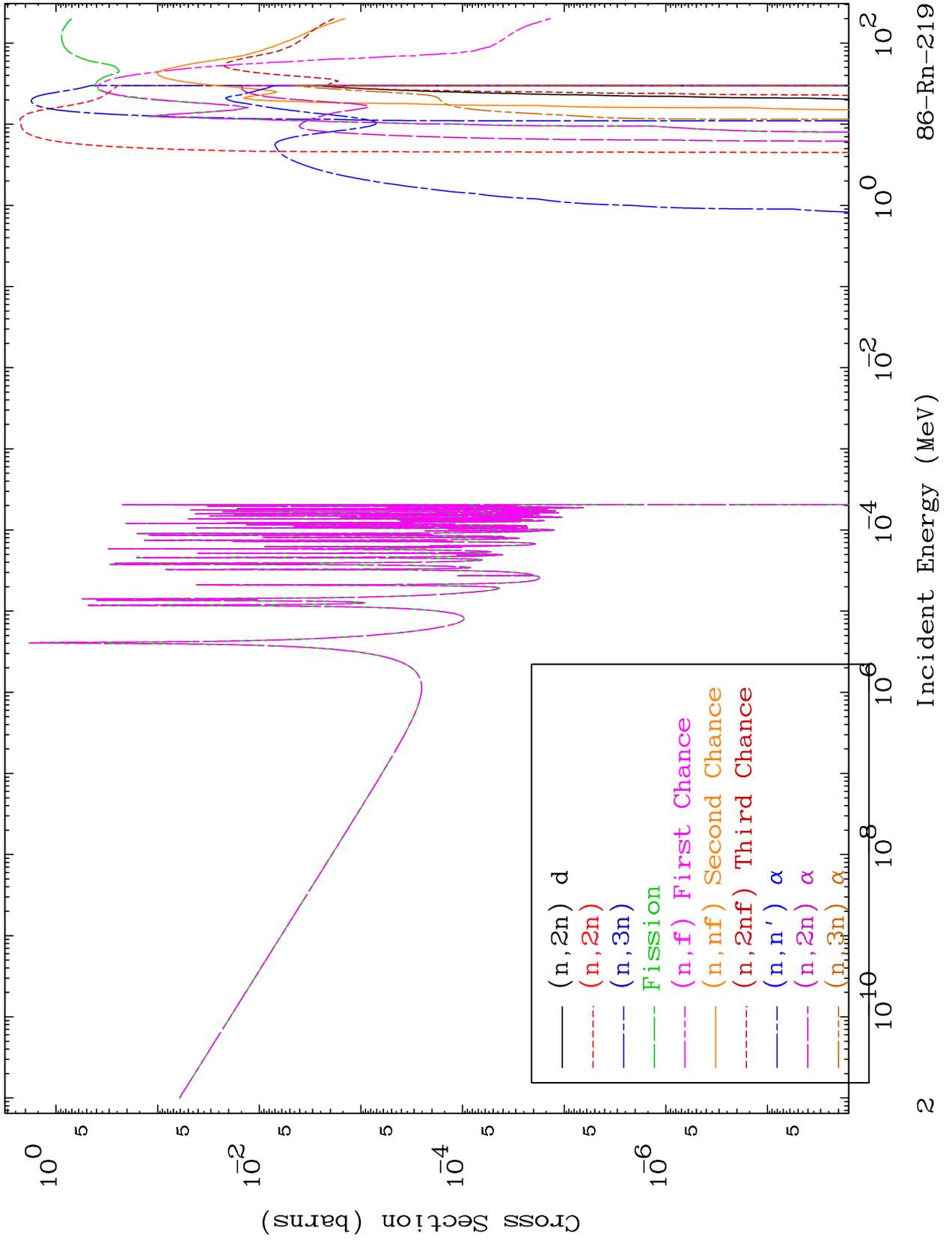


86-Rn-219

MAT 8649

Neutron Absorption
293 Kelvin Cross Sections

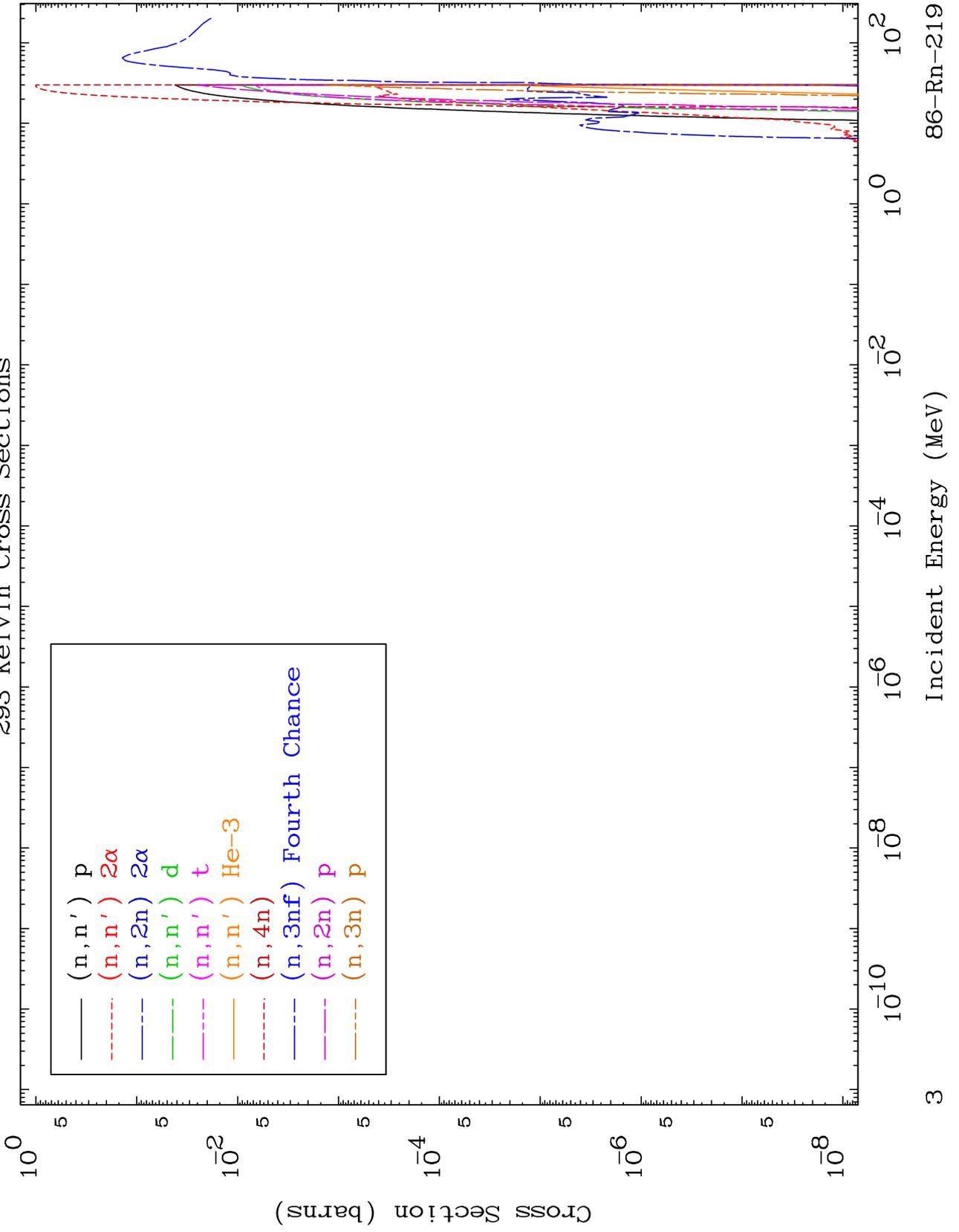
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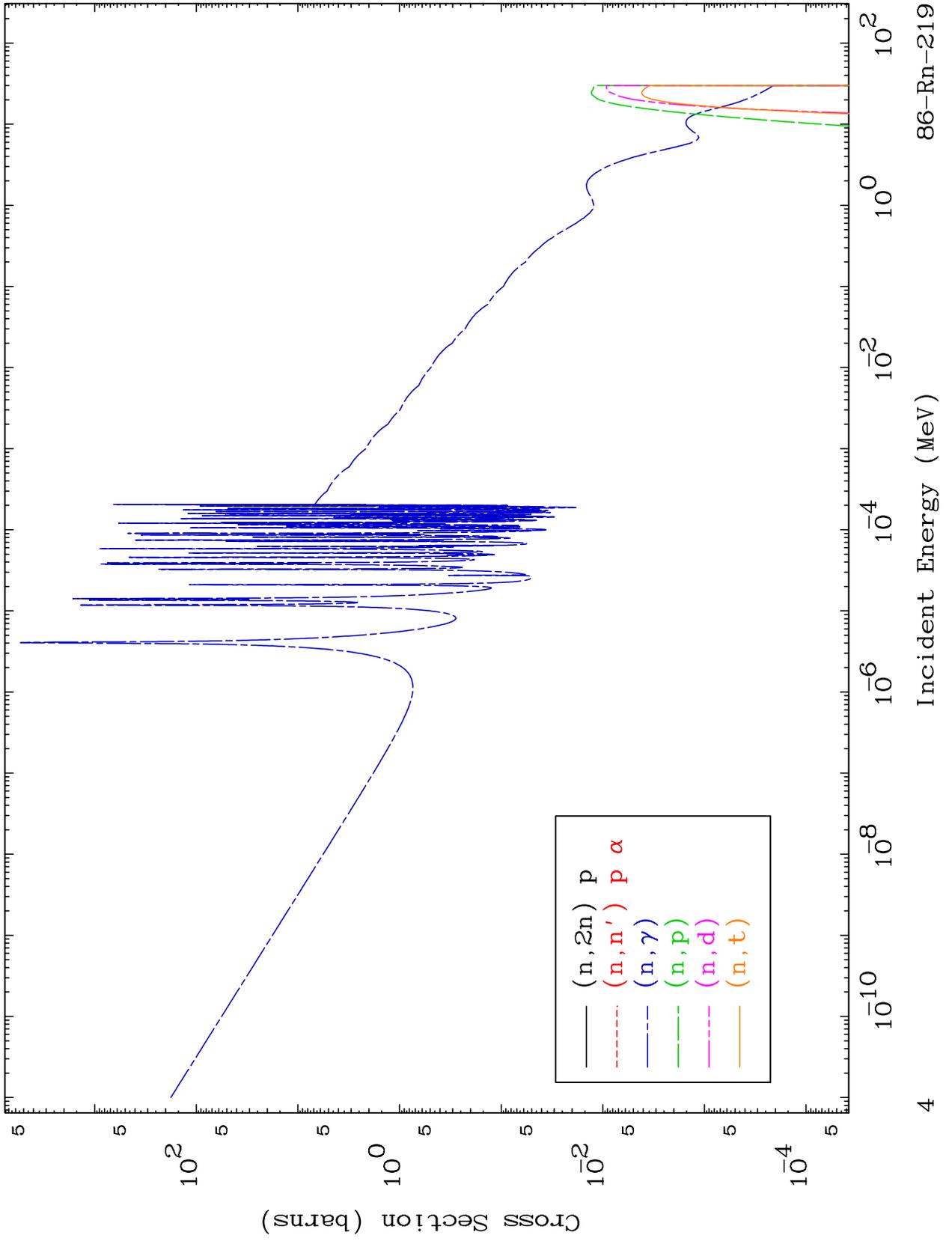


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Neutron Absorption
293 Kelvin Cross Sections

86-Rn-219

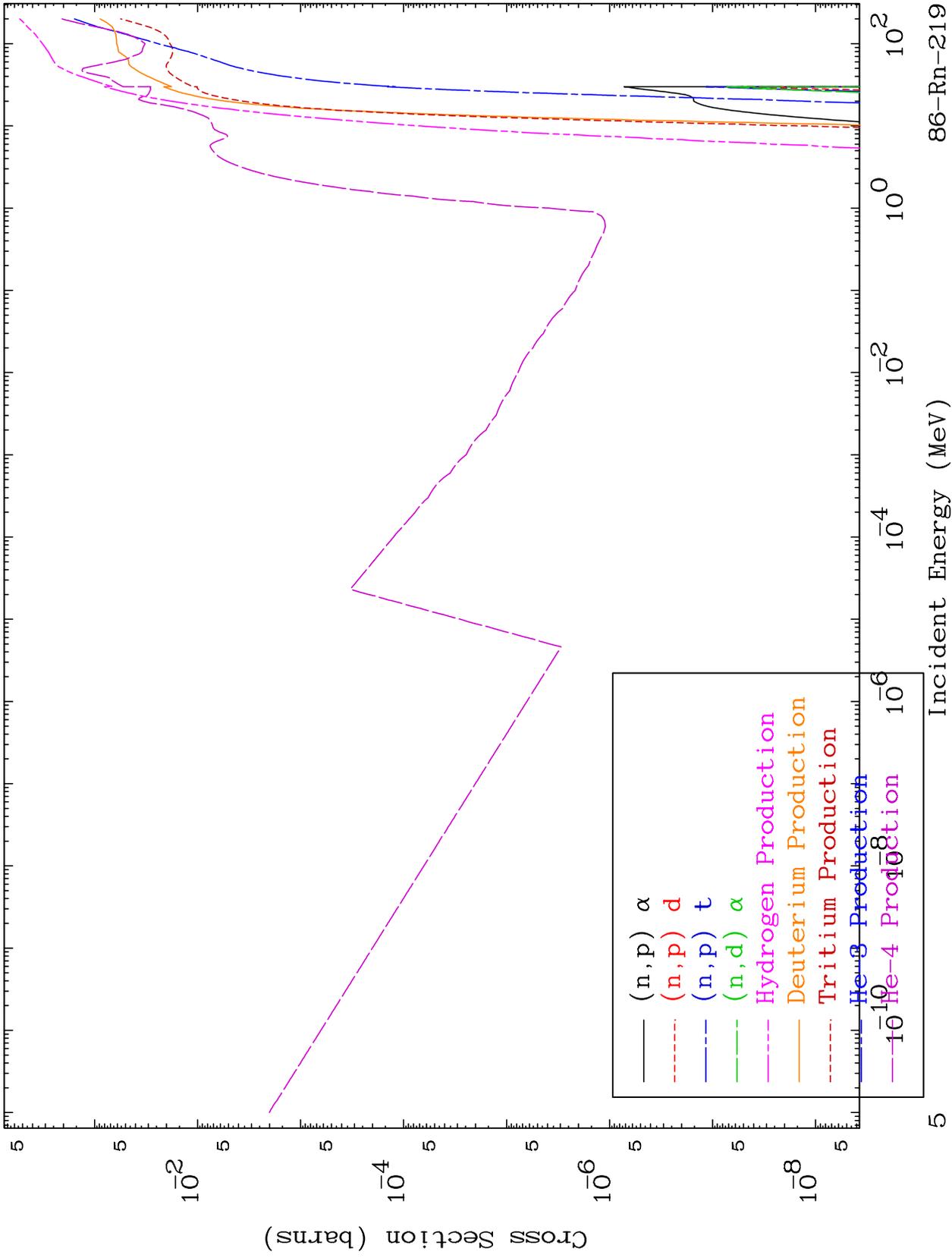




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Neutron Absorption
293 Kelvin Cross Sections

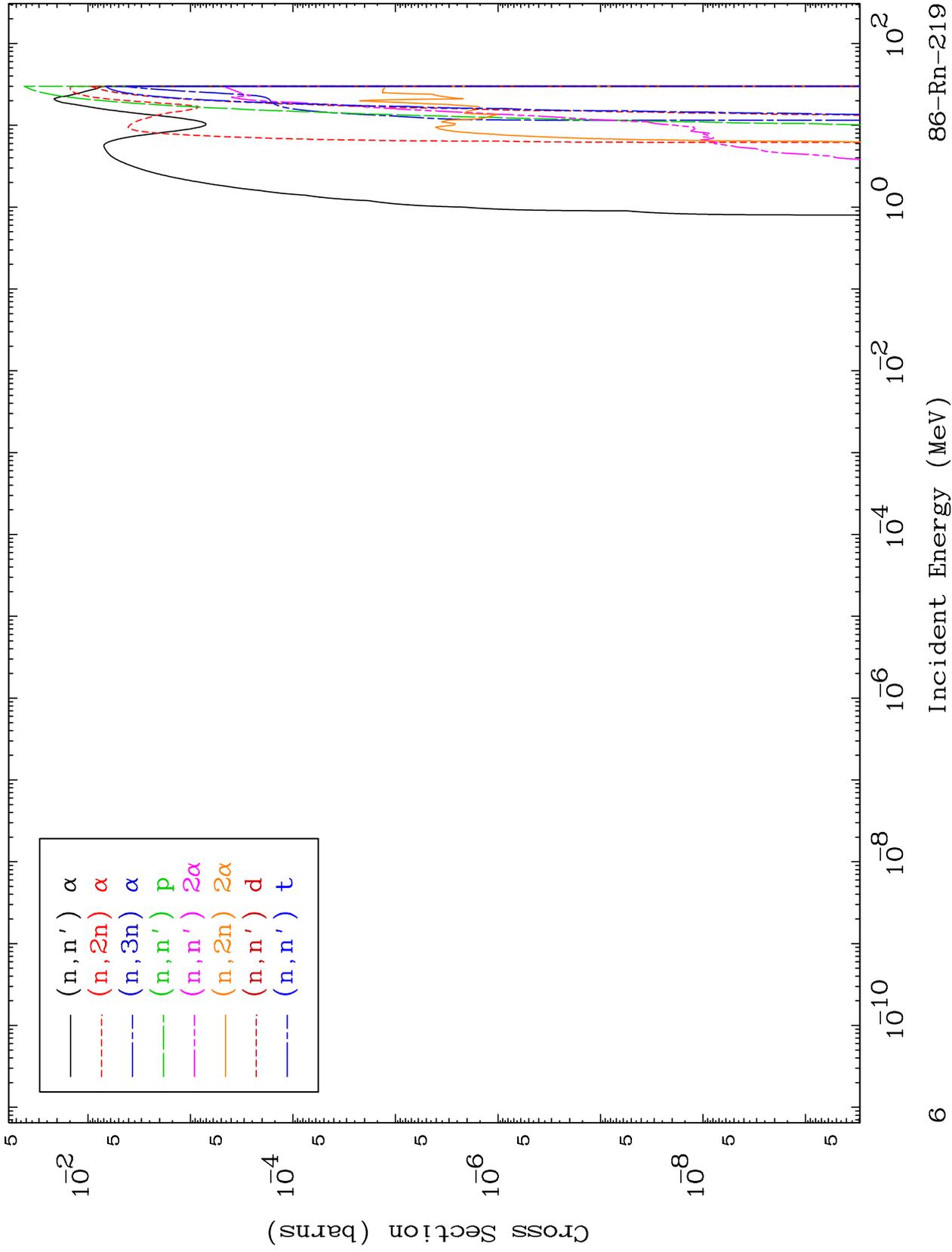
86-Rn-219



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Charged Particle
293 Kelvin Cross Sections

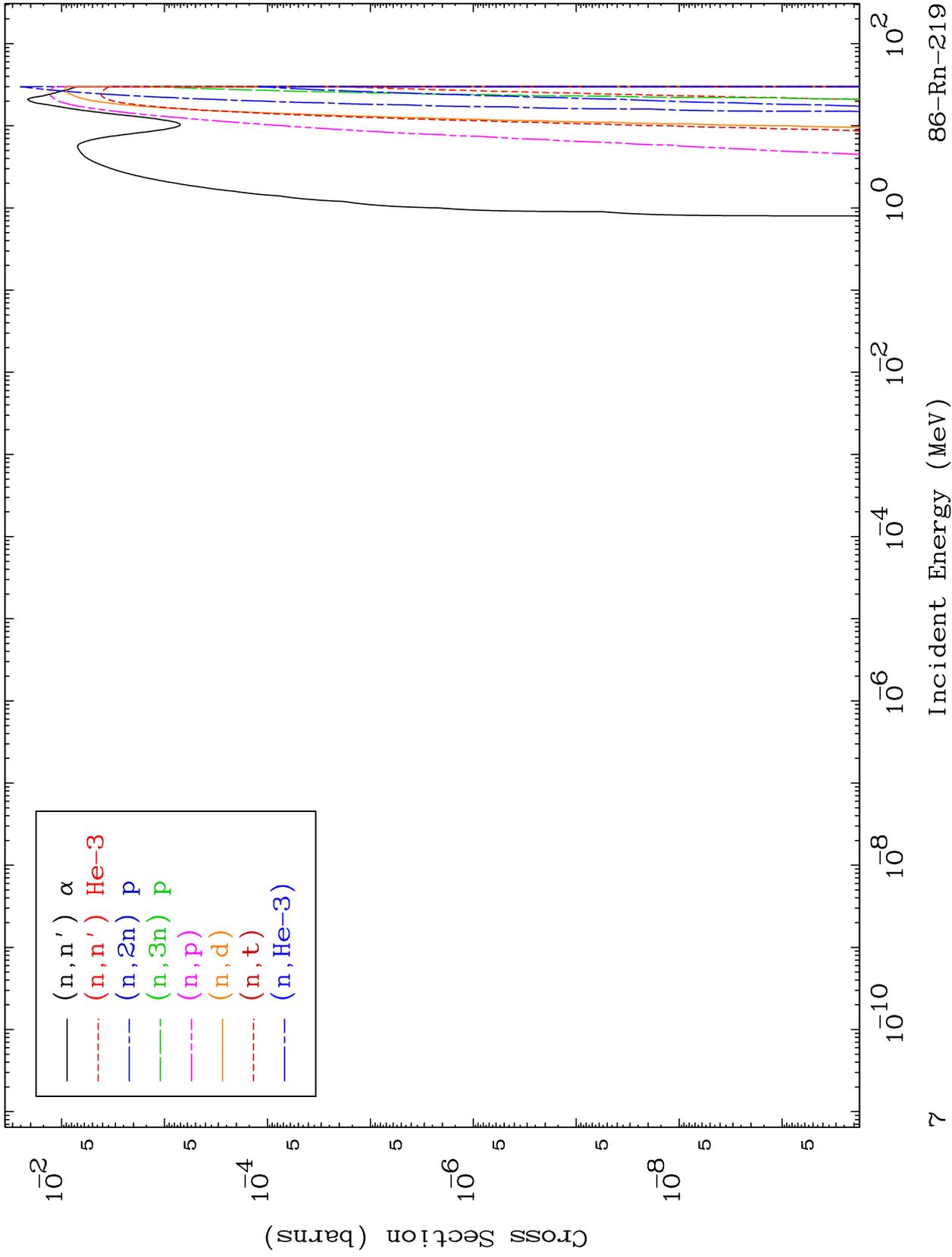
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Charged Particle
293 Kelvin Cross Sections

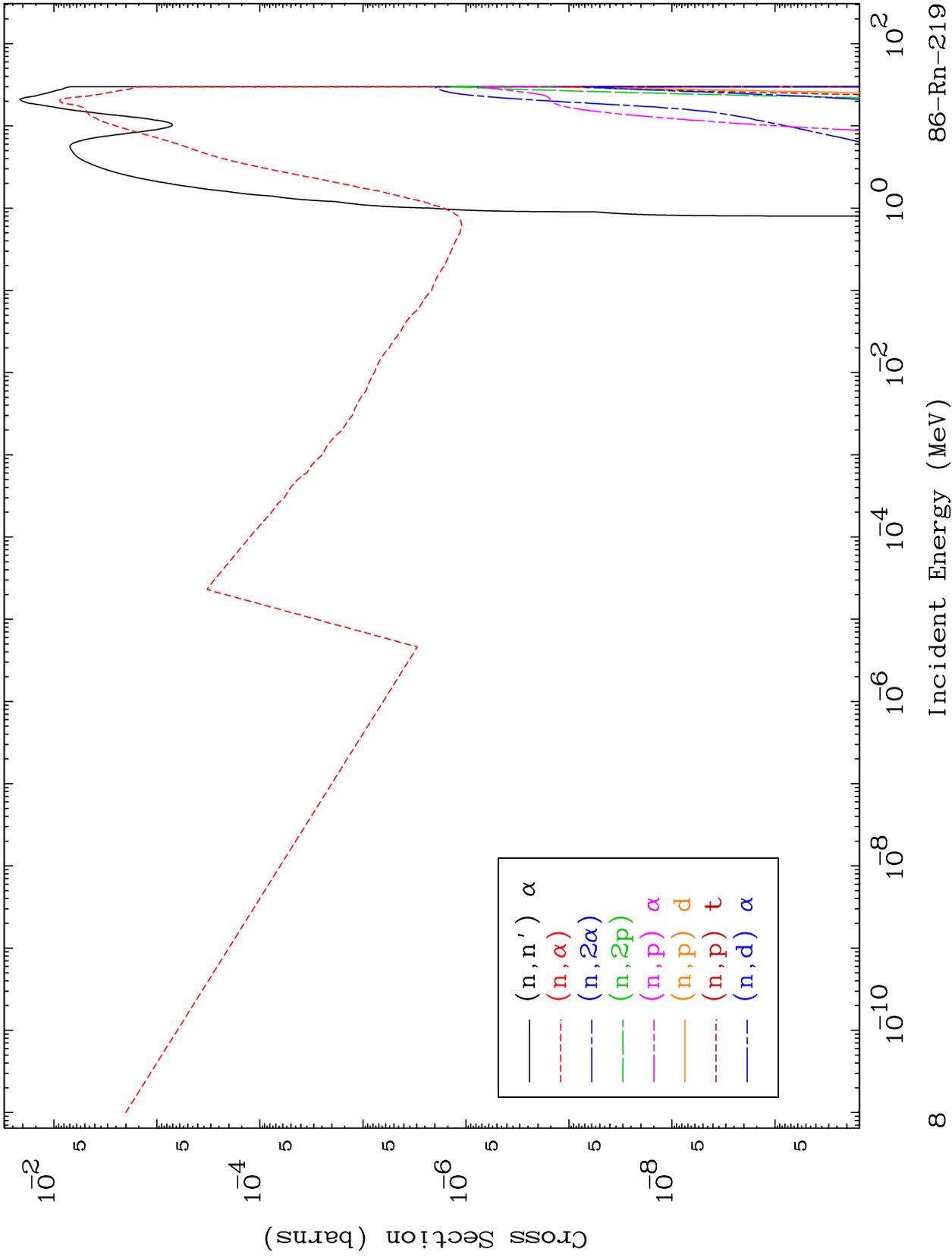
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Charged Particle
293 Kelvin Cross Sections

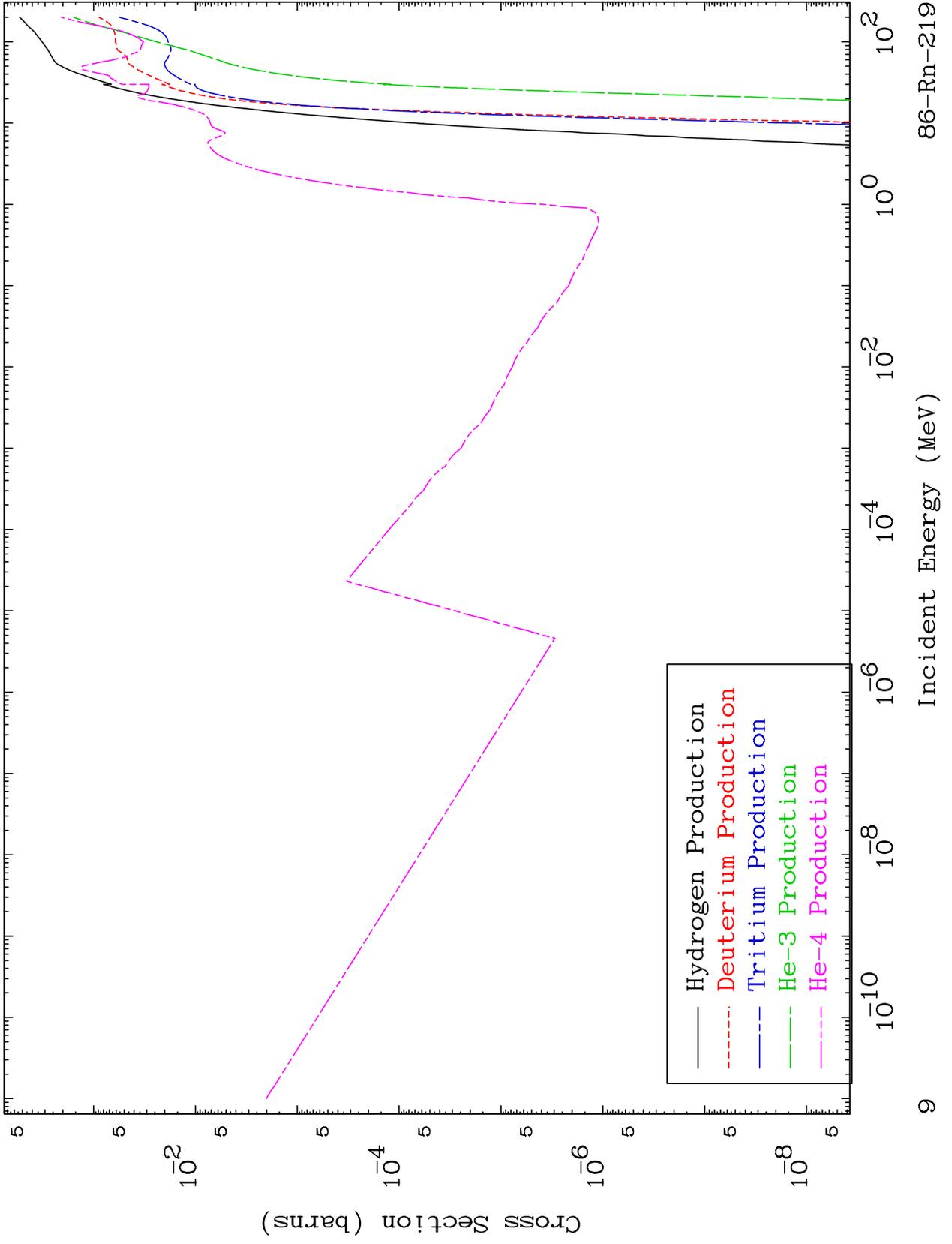
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Particle Production
293 Kelvin Cross Sections

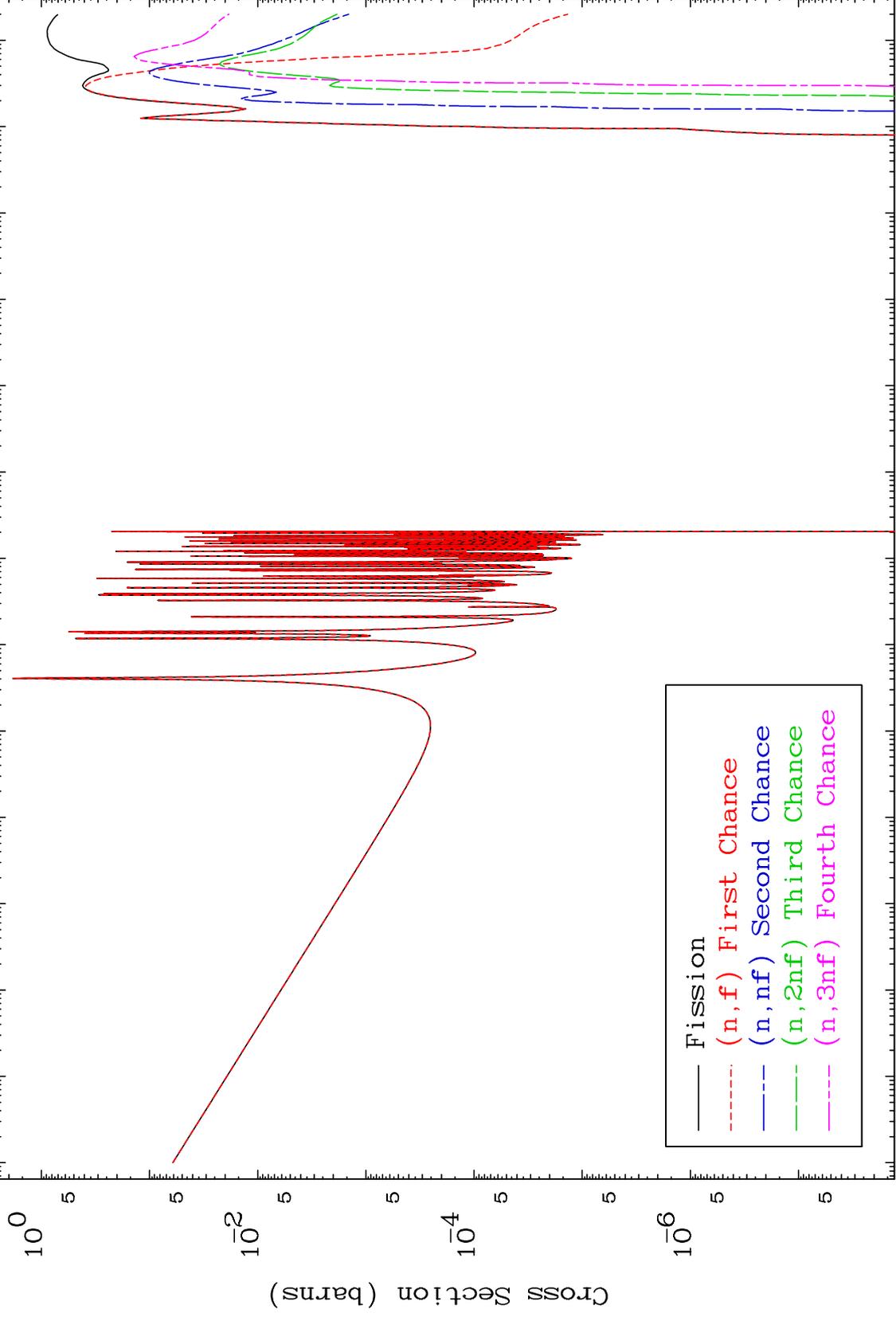
86-Rn-219



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Fission
293 Kelvin Cross Sections

86-Rn-219



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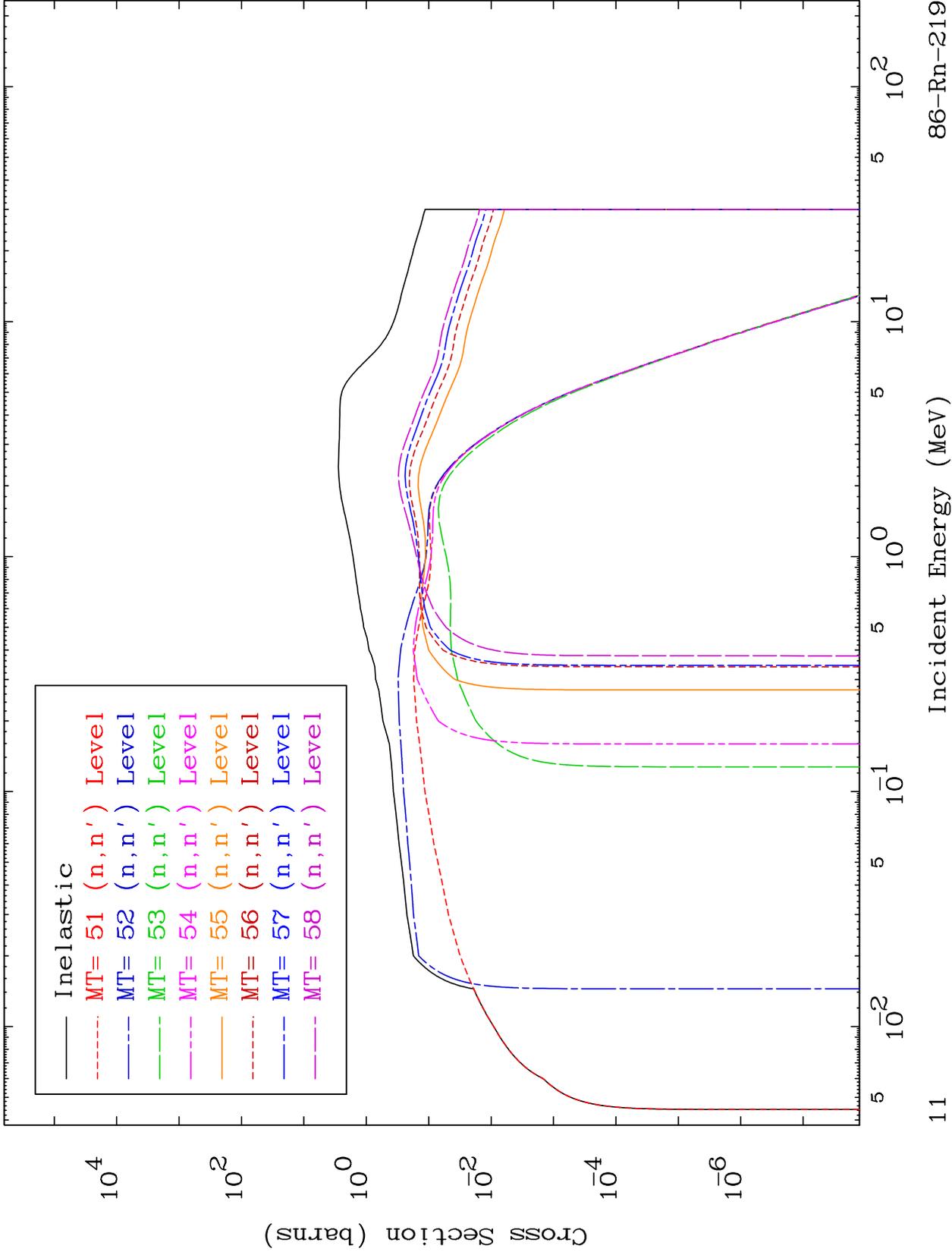
Incident Energy (MeV)

86-Rn-219

MAT 8649

(n,n') Levels
293 Kelvin Cross Sections

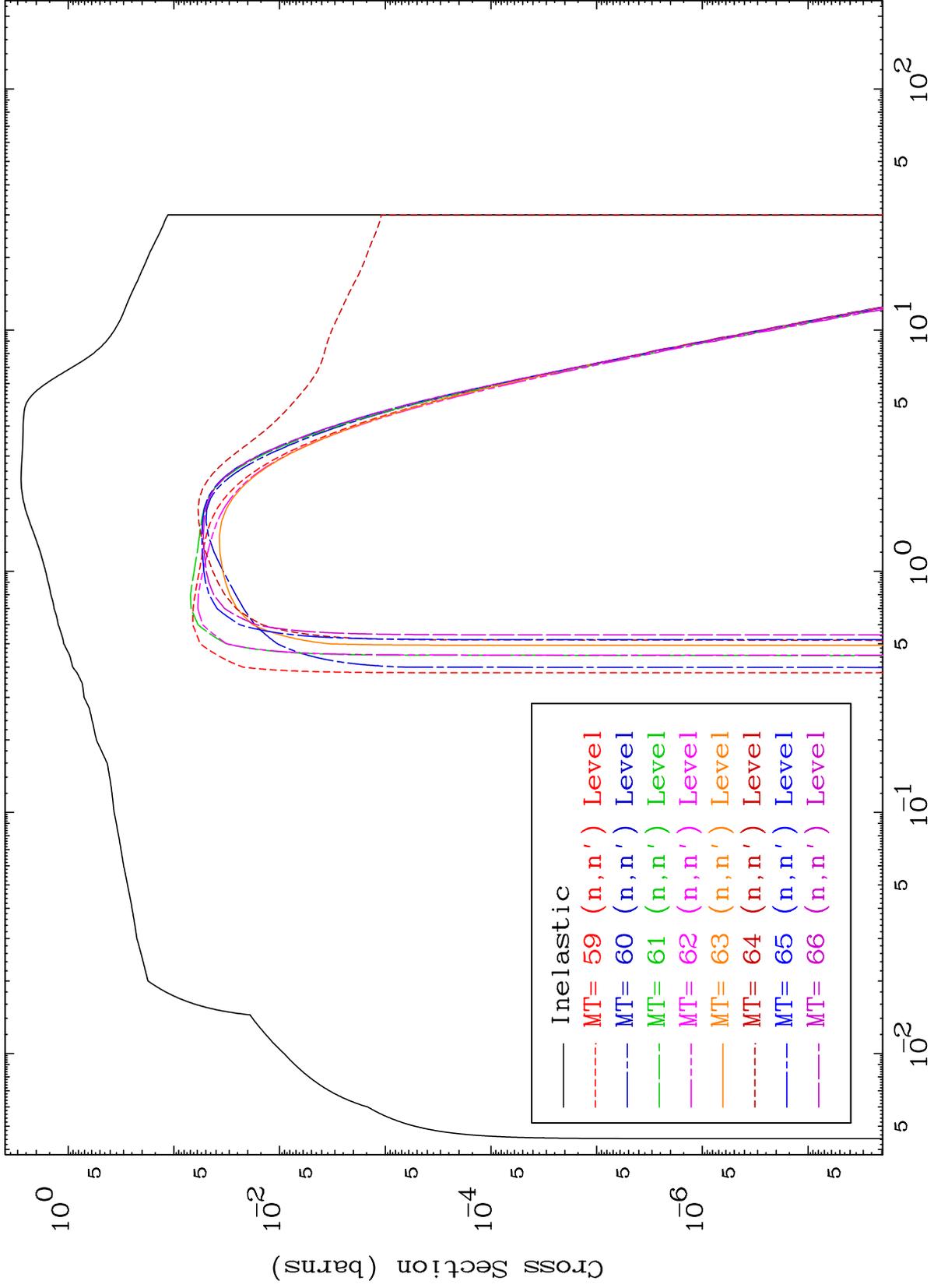
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MAT 8649

(n,n') Levels
293 Kelvin Cross Sections

86-Rn-219



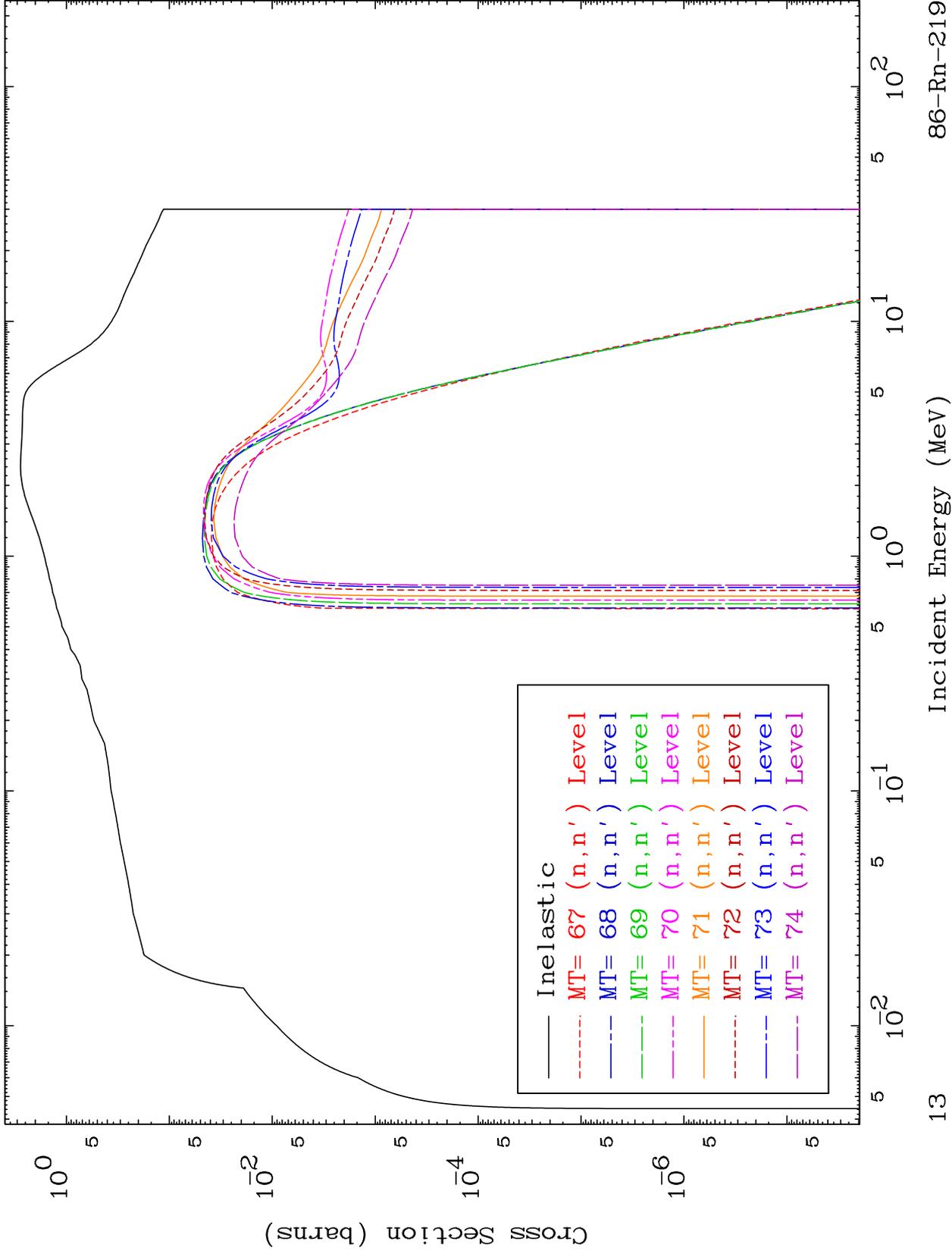
12

86-Rn-219

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(n,n') Levels
293 Kelvin Cross Sections

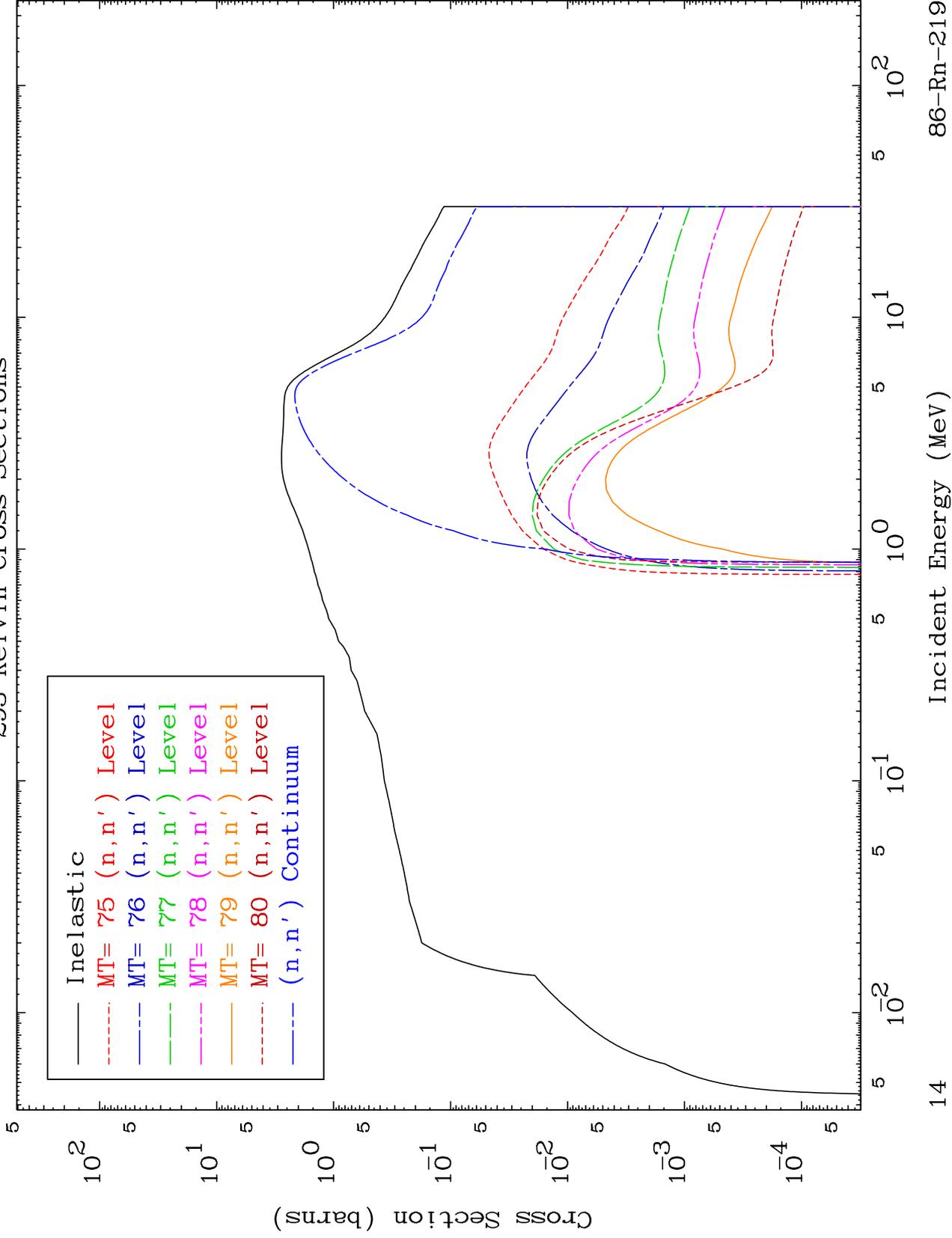
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13

Incident Energy (MeV)

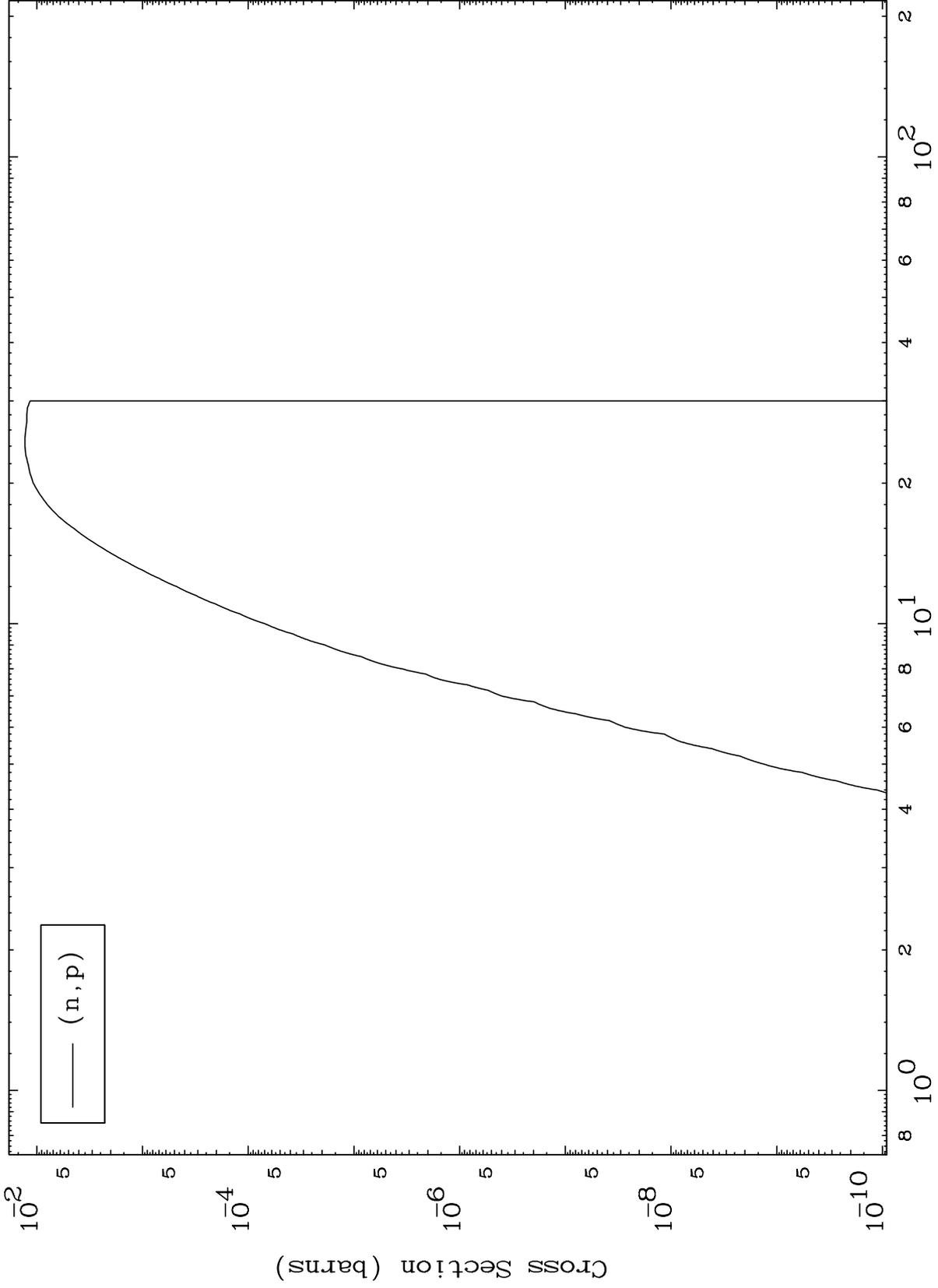
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(n,p) Levels
293 Kelvin Cross Sections

86-Rn-219



15

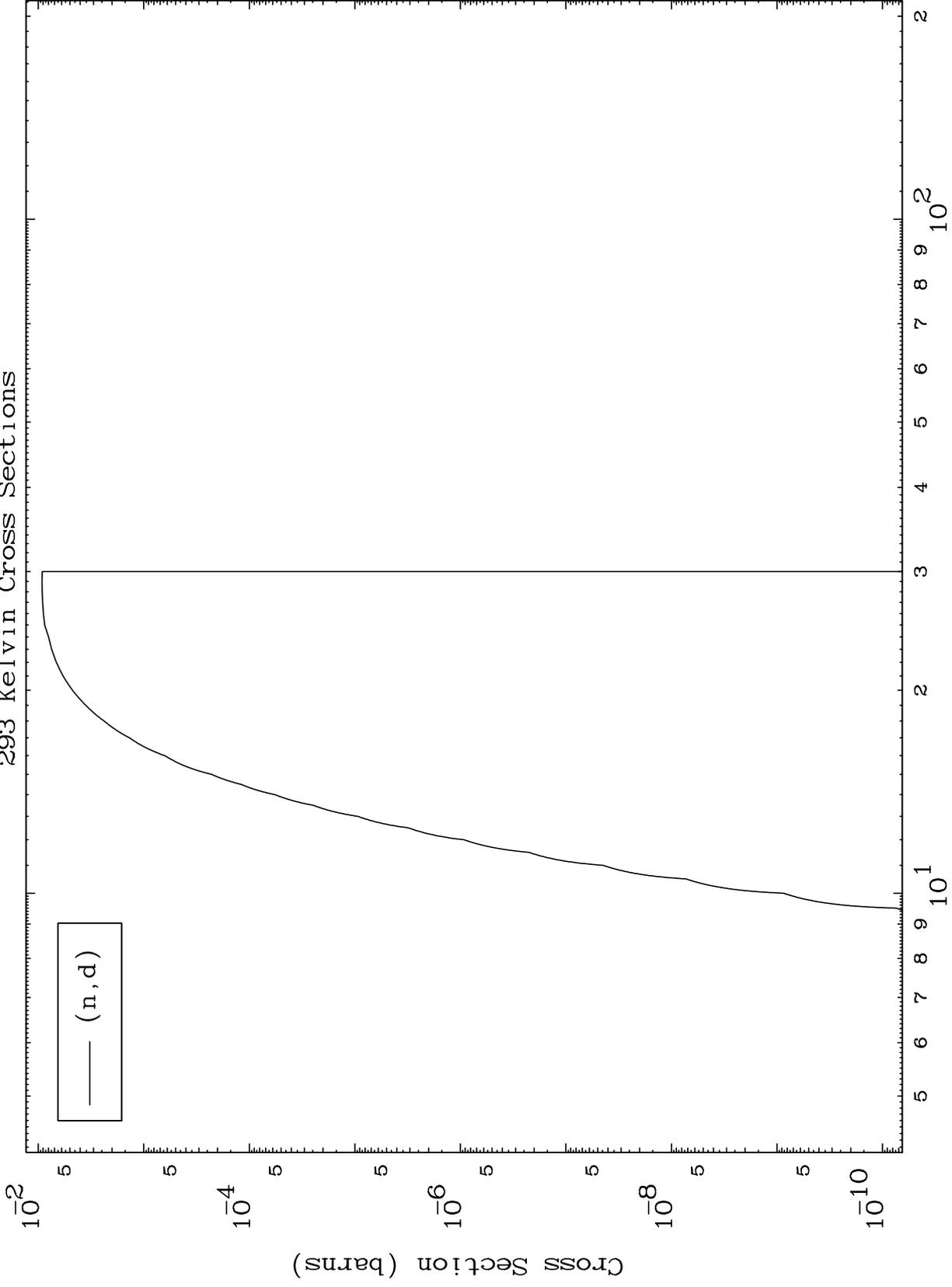
Incident Energy (MeV)

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MAT 8649

(n,d) Levels
293 Kelvin Cross Sections

86-Rn-219



16

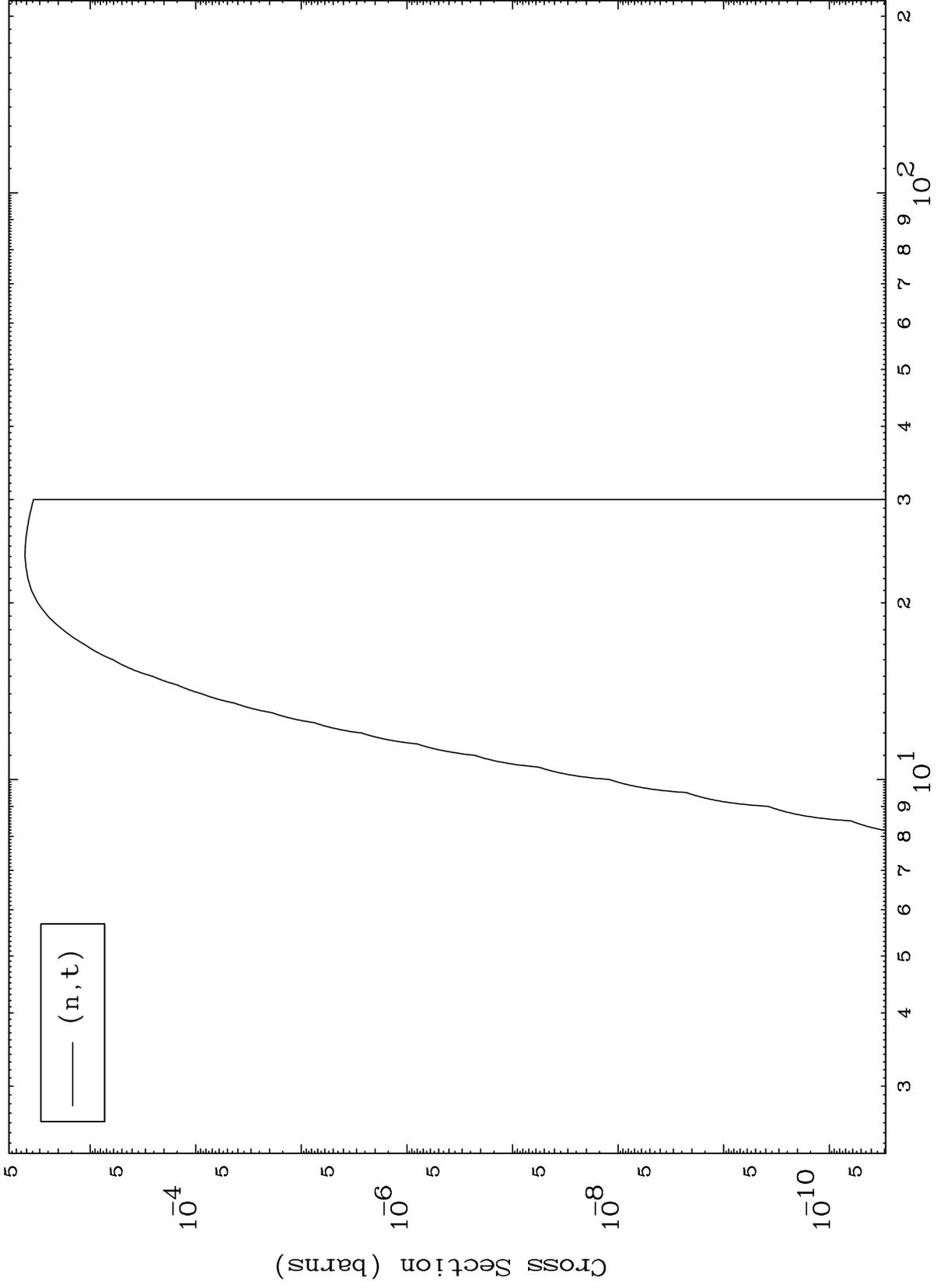
Incident Energy (MeV)

86-Rn-219

MAT 8649

(n,t) Levels
293 Kelvin Cross Sections

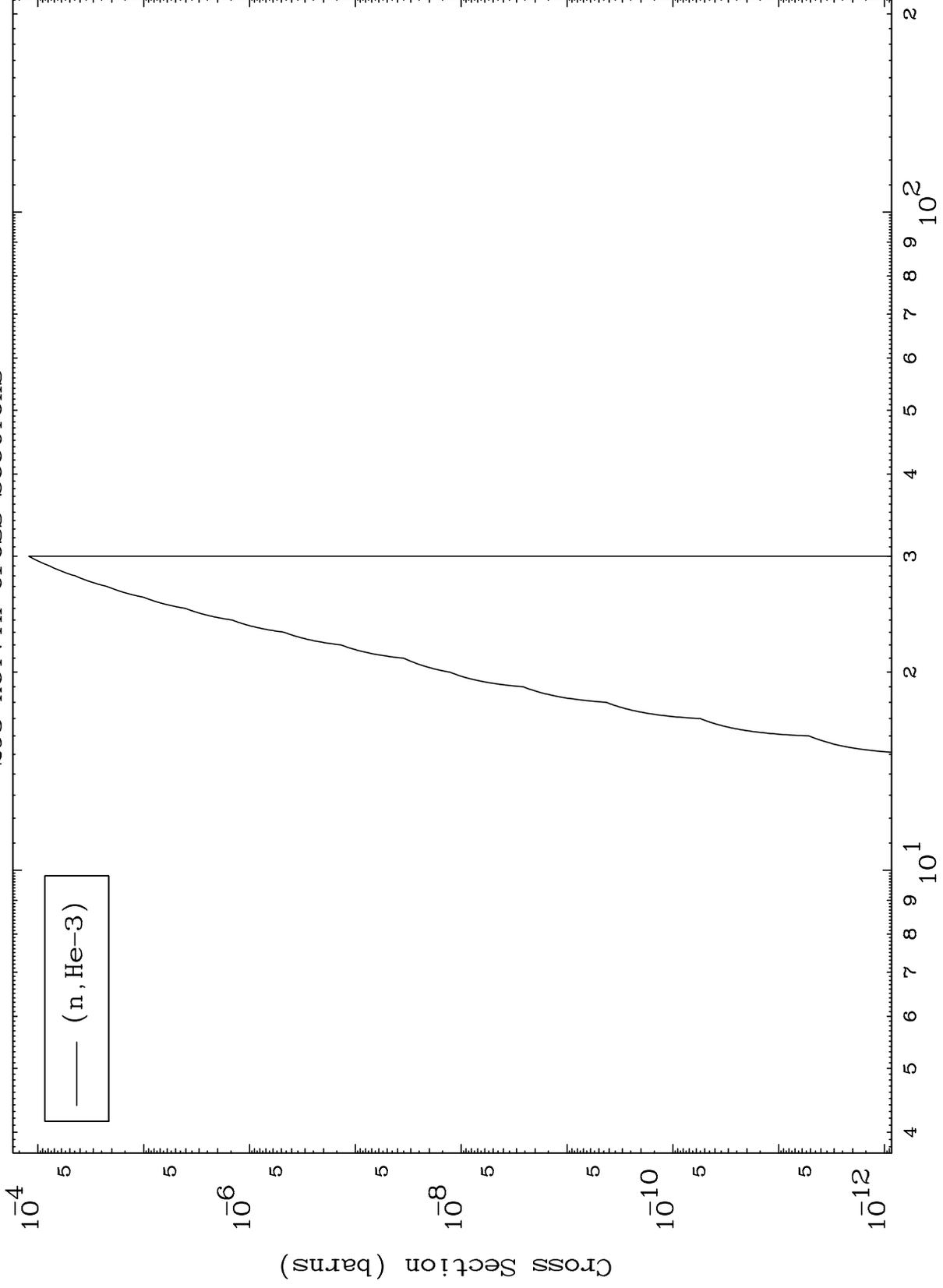
86-Rn-219



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(n,He3) Levels
293 Kelvin Cross Sections

86-Rn-219



18

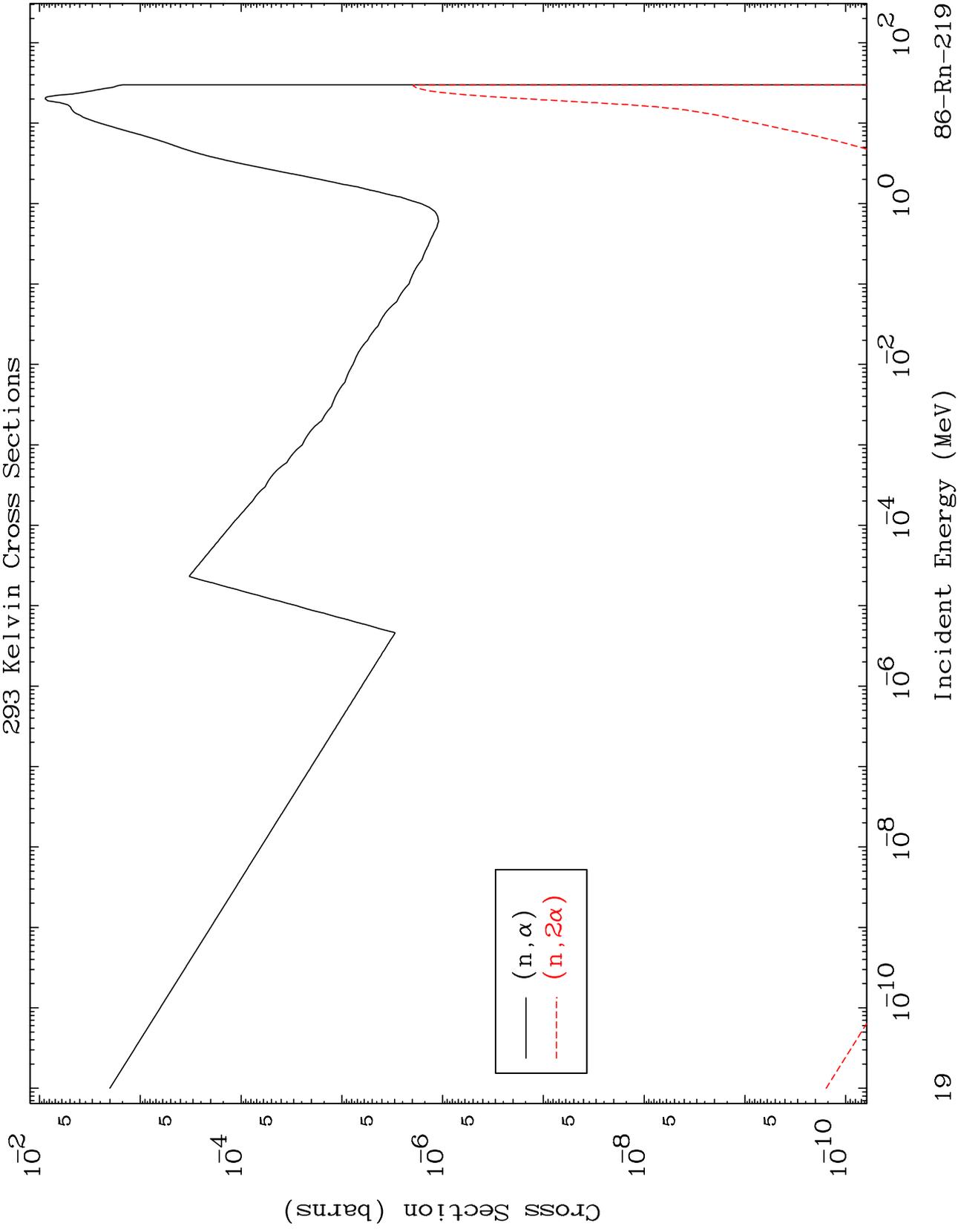
Incident Energy (MeV)

86-Rn-219

MAT 8649

(n, α) Levels
293 Kelvin Cross Sections

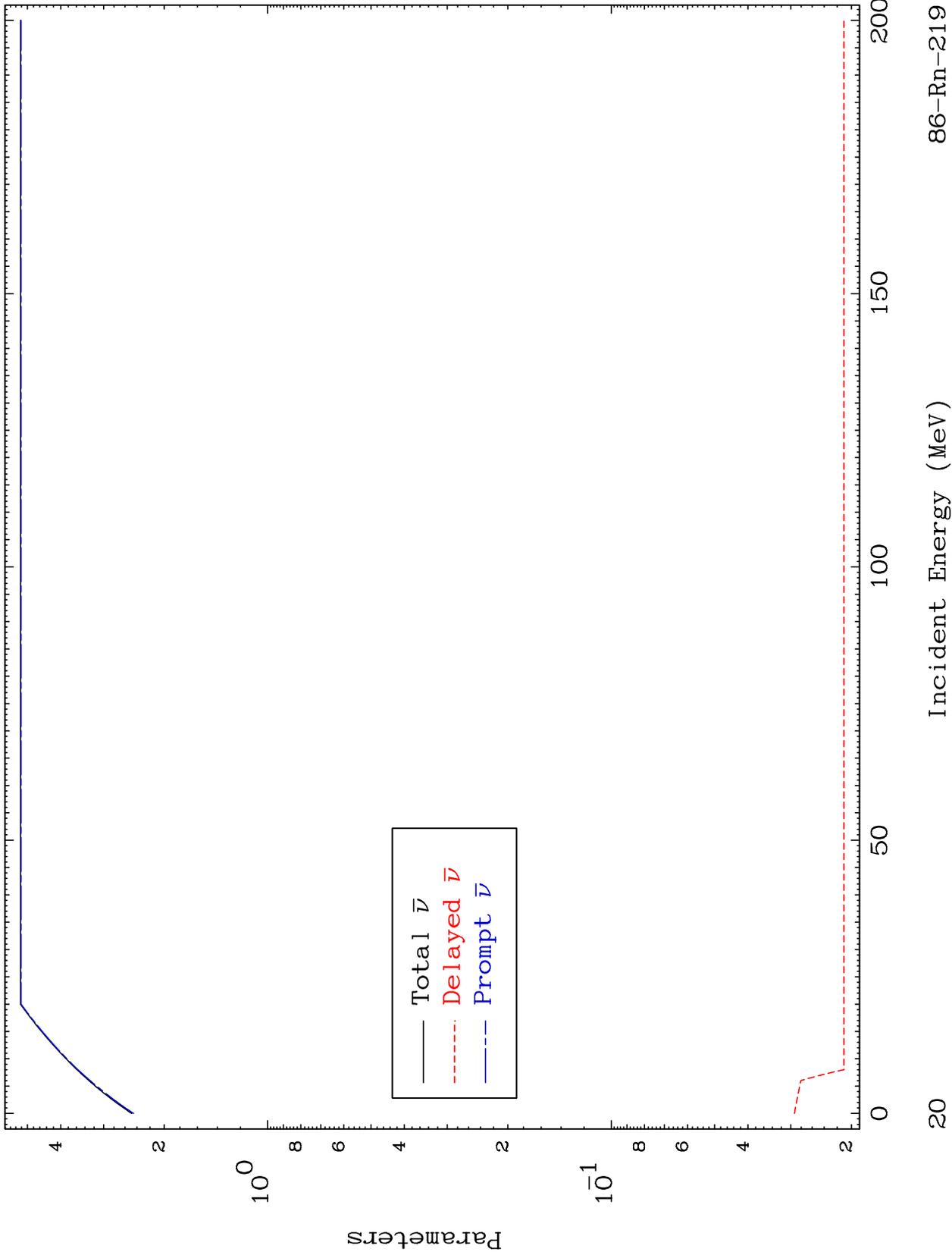
86-Rn-219



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Energy Release
Parameters

86-Rn-219



86-Rn-219

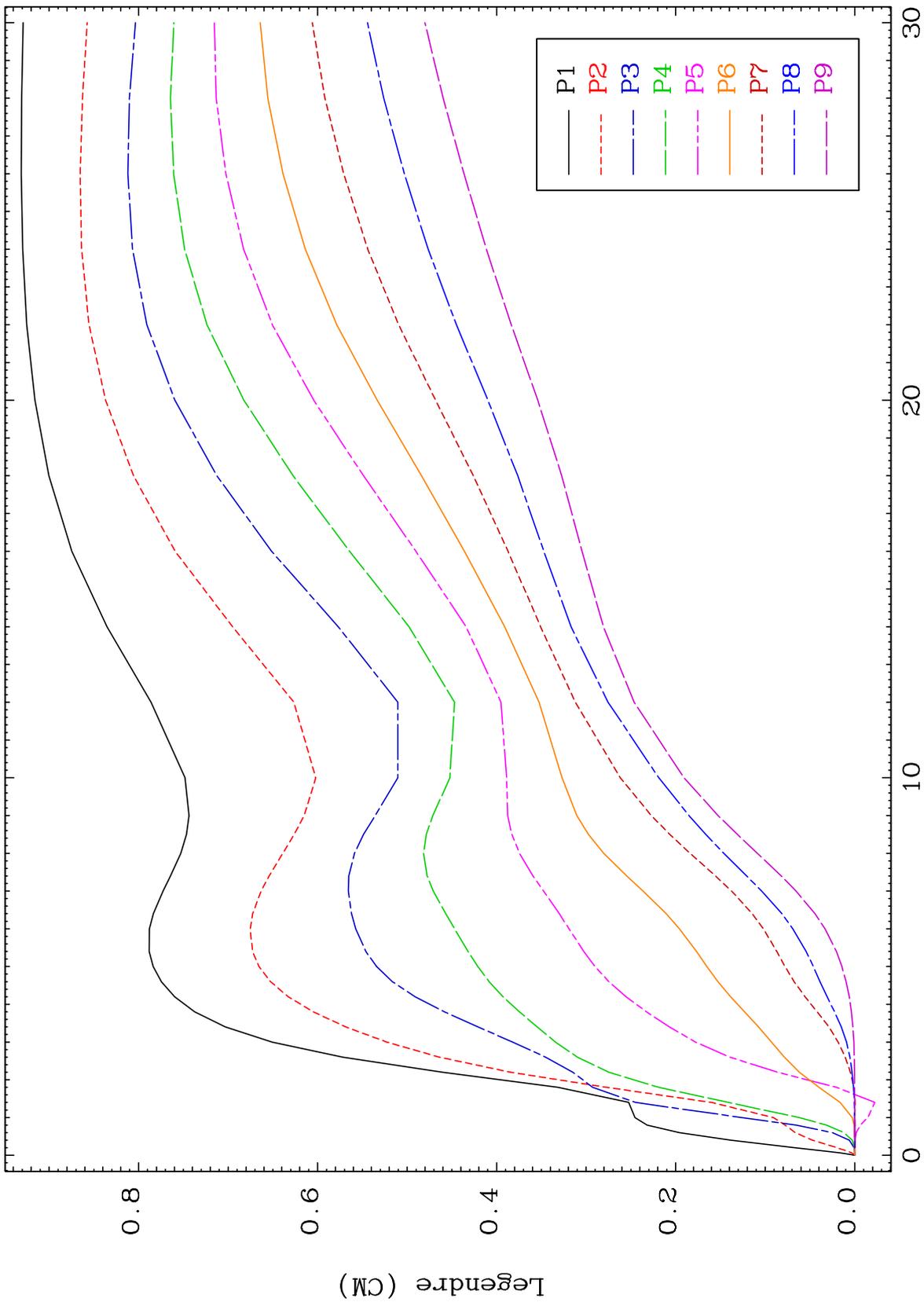
Incident Energy (MeV)

20

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86-Rn-219

Elastic Legendre Coefficients



86-Rn-219

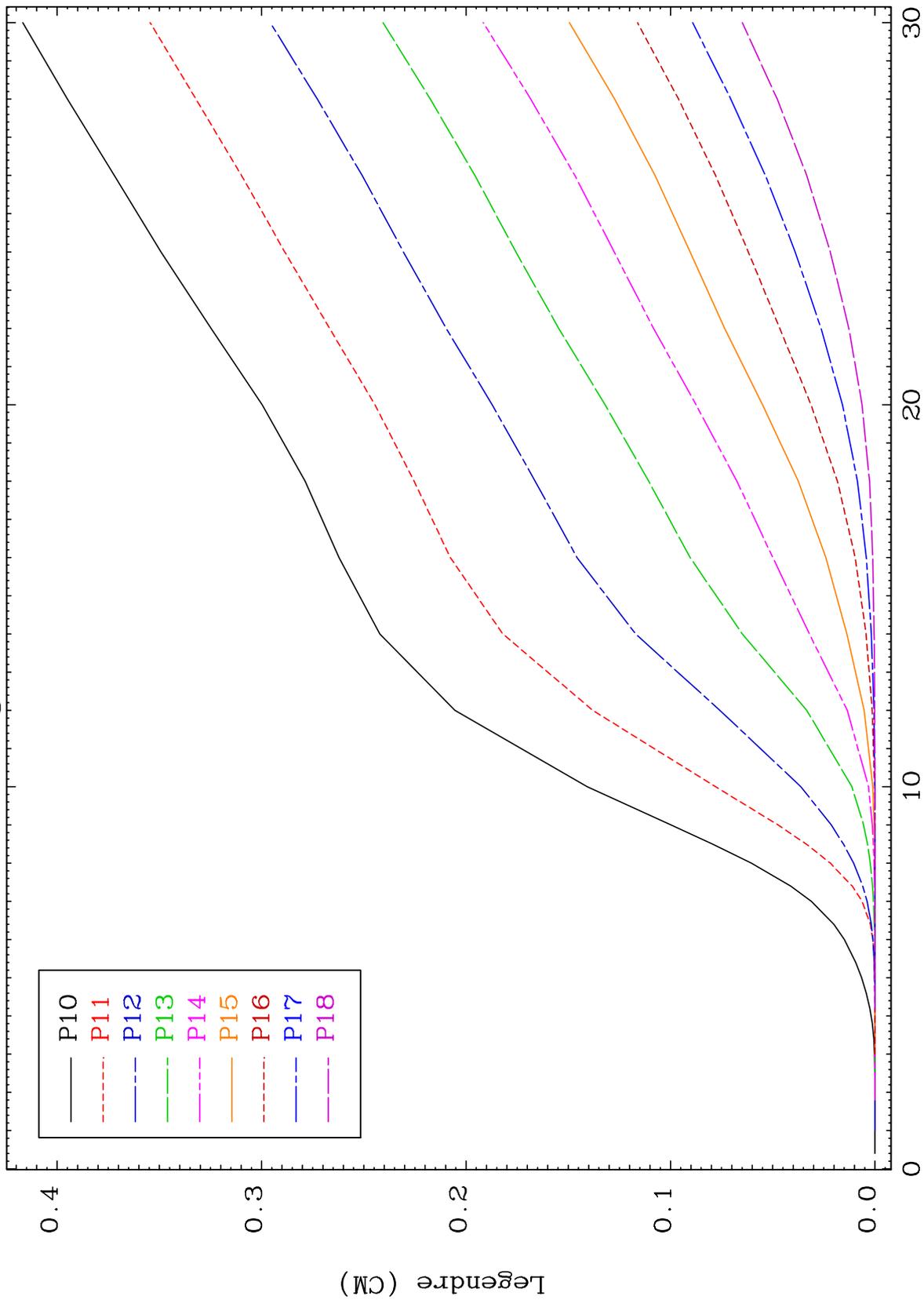
Incident Energy (MeV)

21

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Elastic Legendre Coefficients

86-Rn-219



22

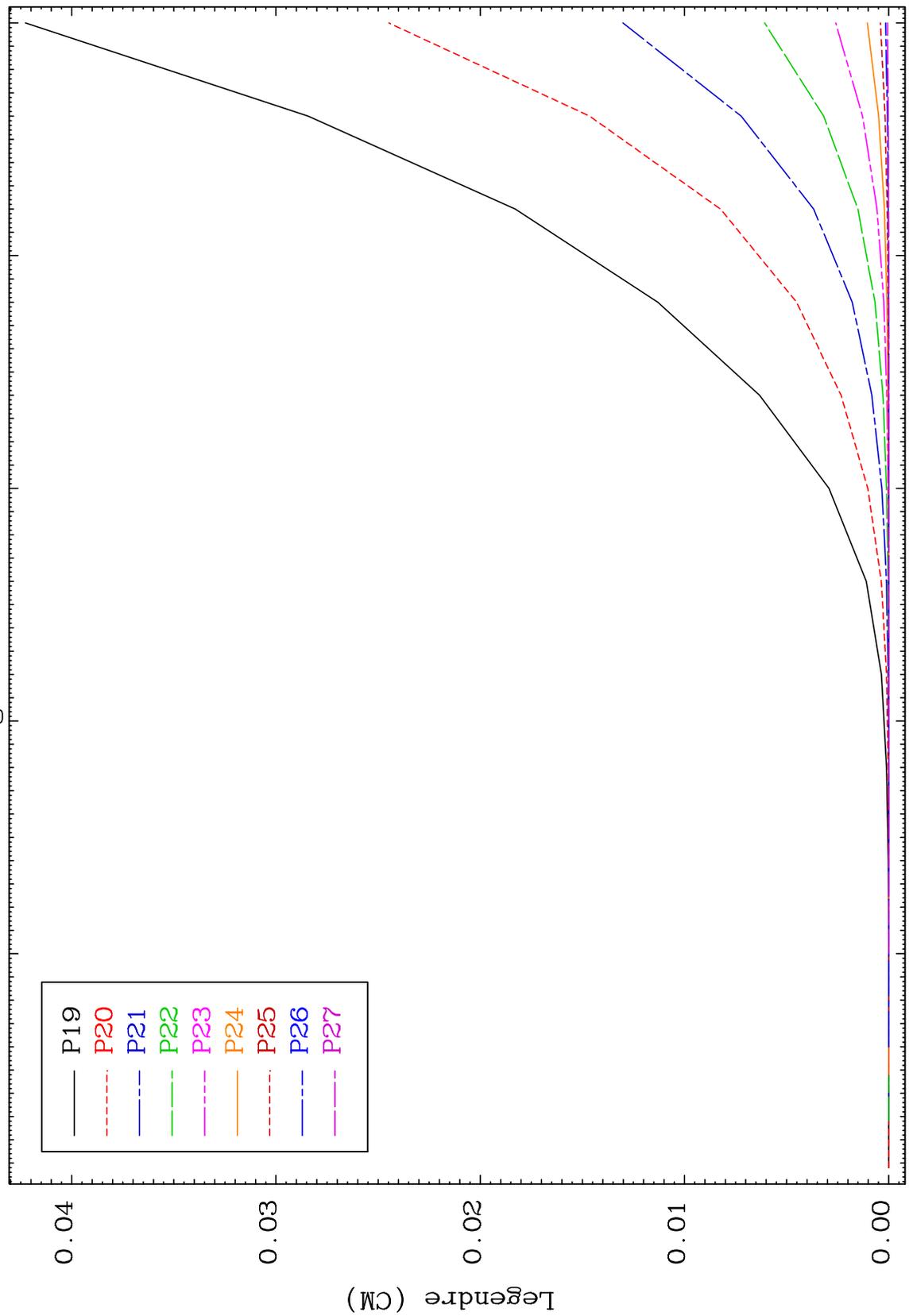
Incident Energy (MeV)

86-Rn-219

MAT 8649

Elastic Legendre Coefficients

86-Rn-219



23

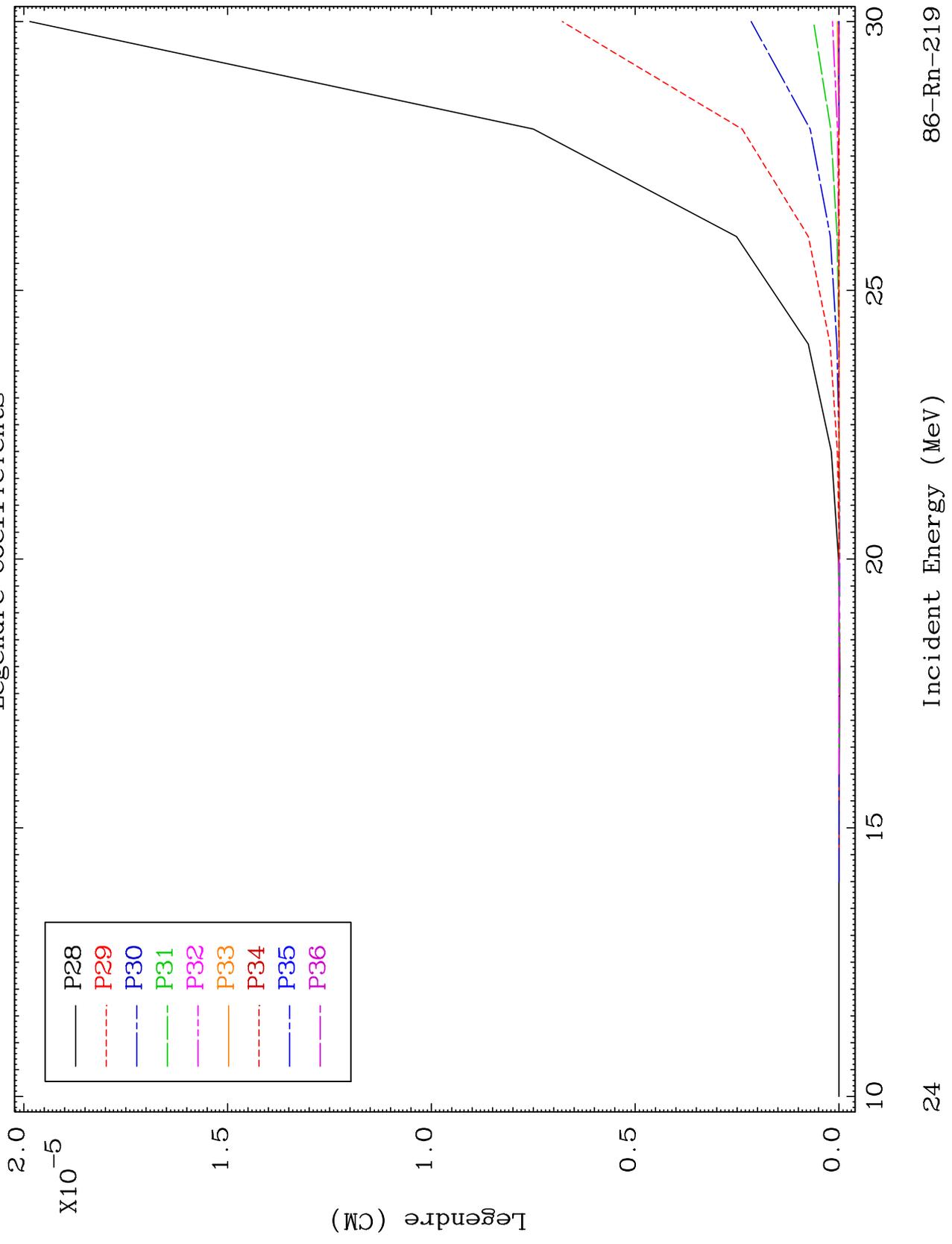
Incident Energy (MeV)

86-Rn-219

MAT 8649

Elastic Legendre Coefficients

86-Rn-219

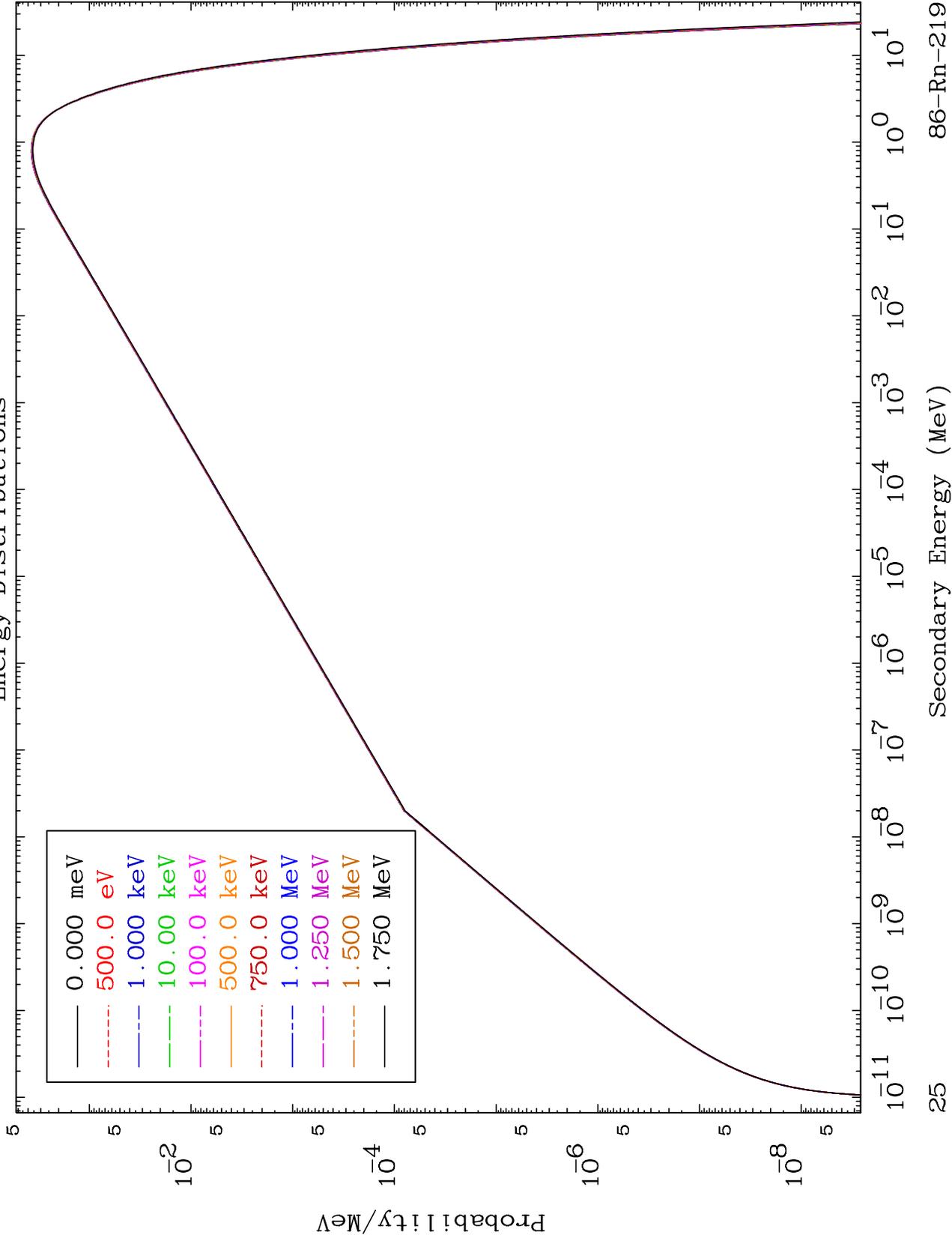


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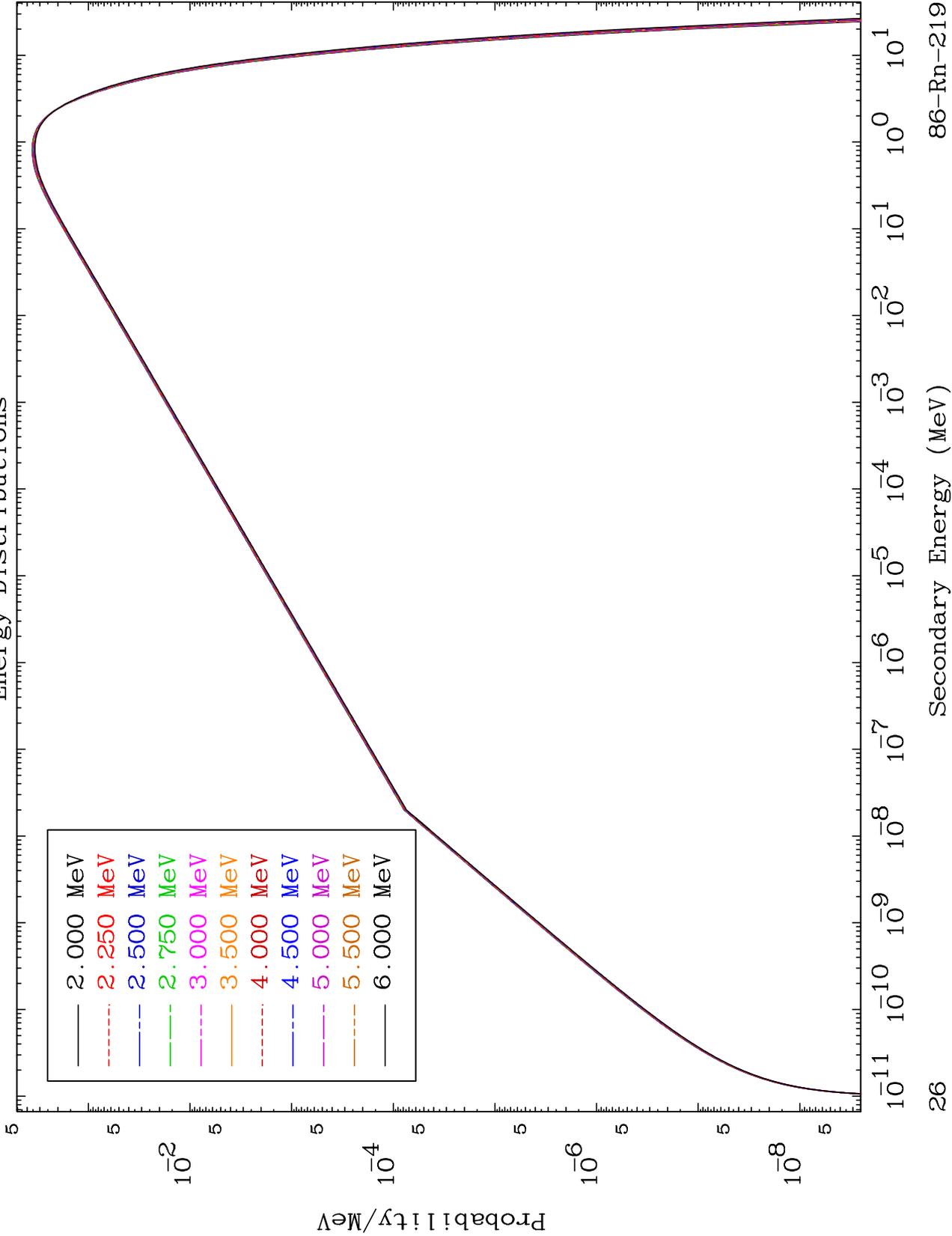
Incident Energy (MeV)

24

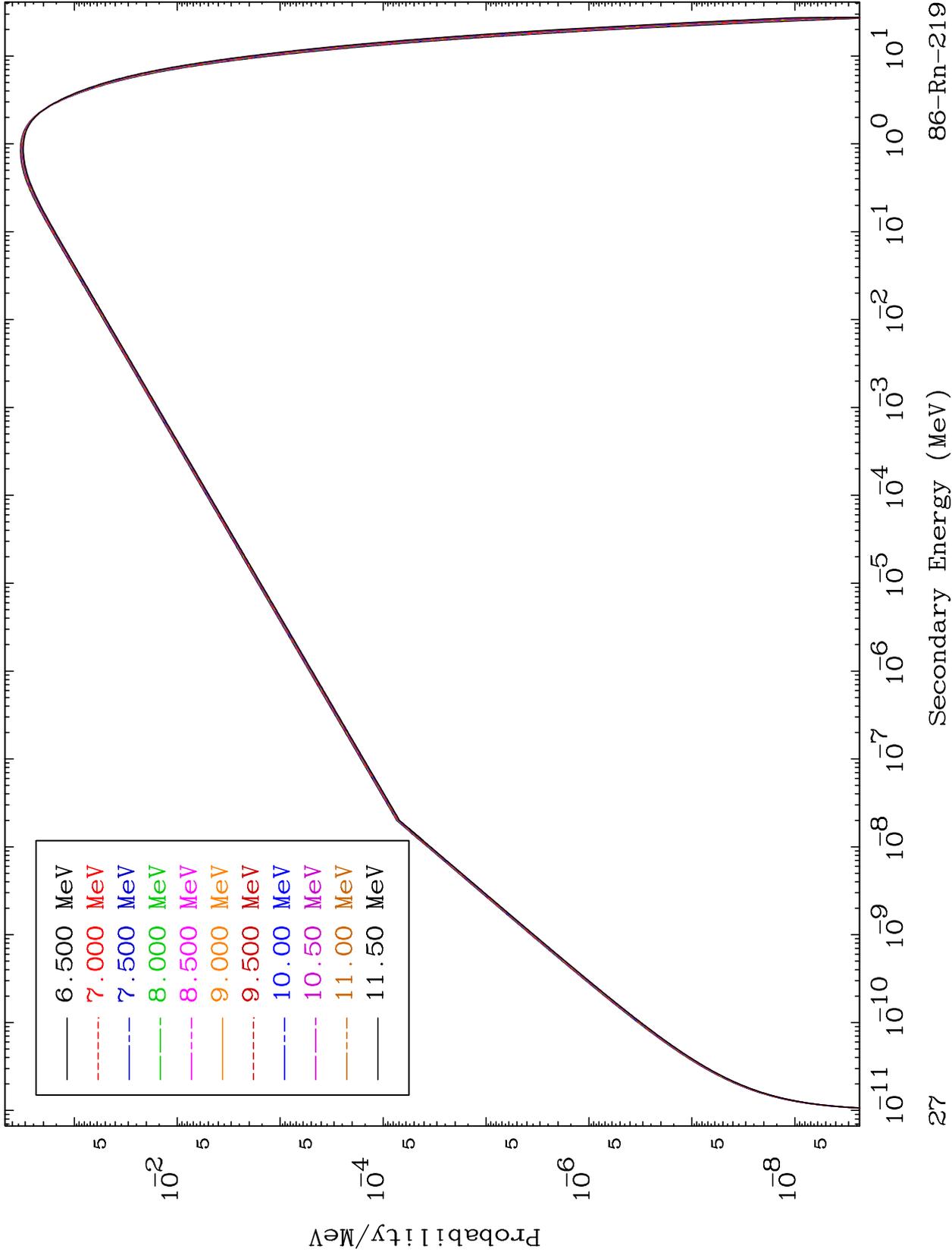
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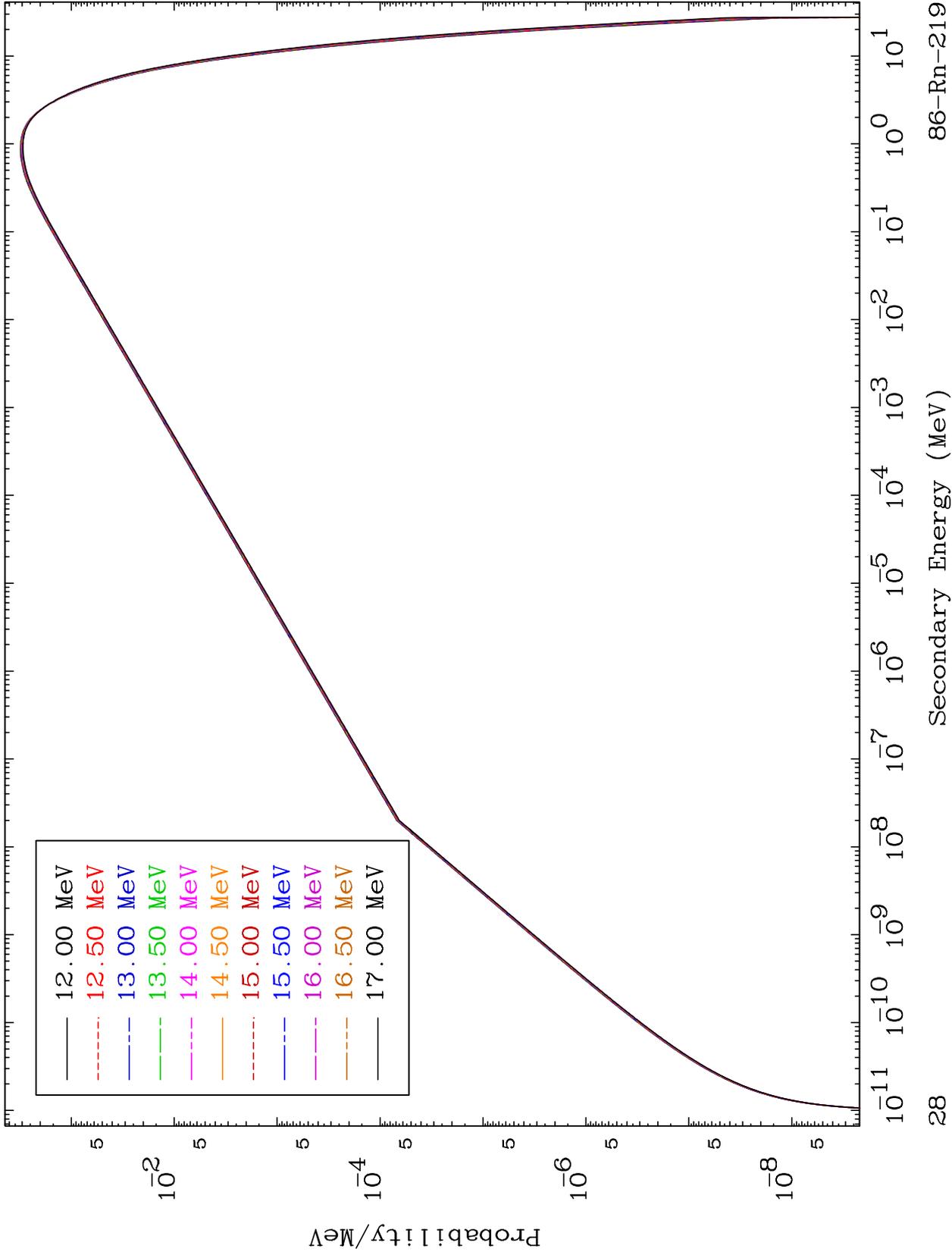
Fission Energy Distributions



Fission Energy Distributions

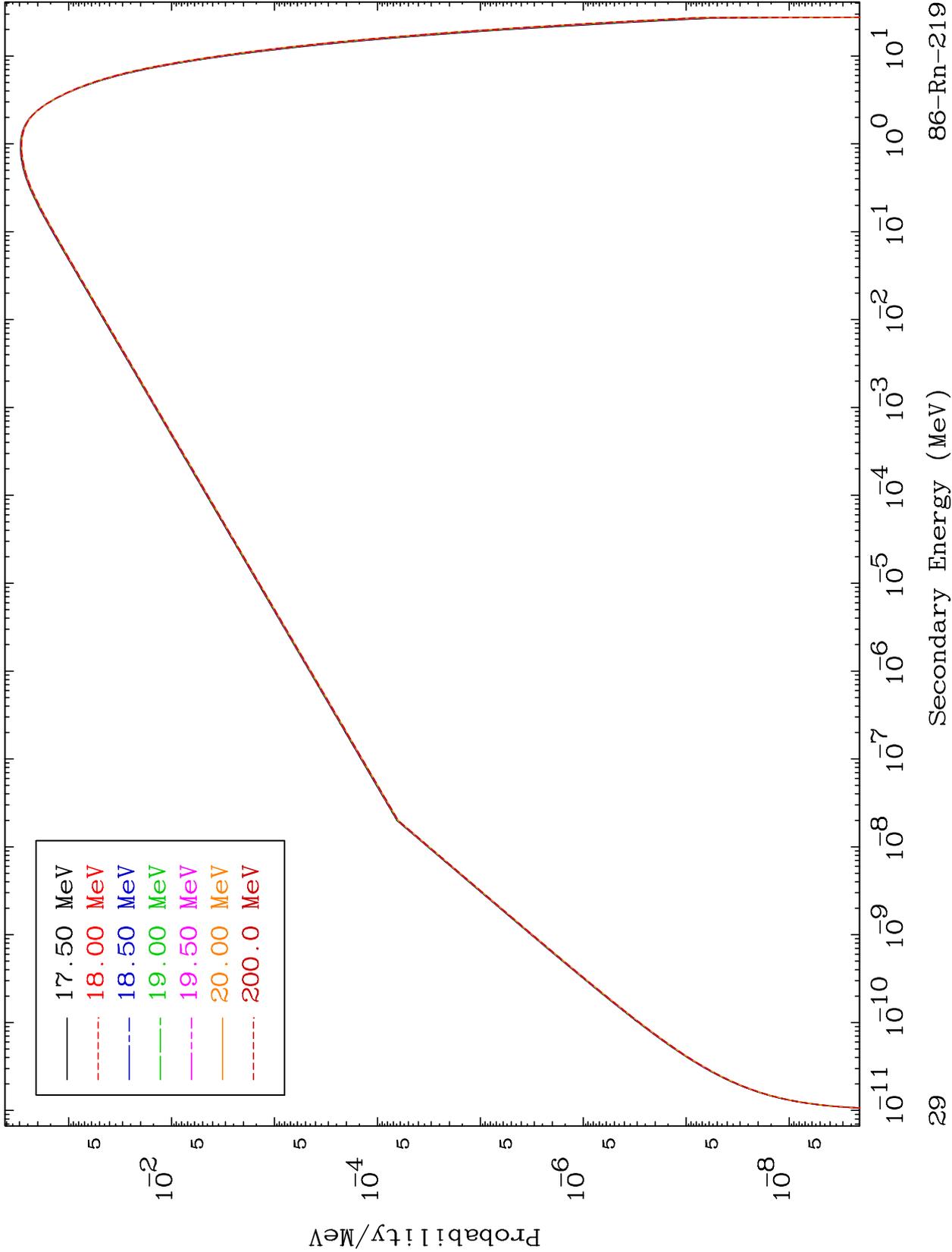


Fission Energy Distributions



Fission Energy Distributions

Fission Energy Distributions

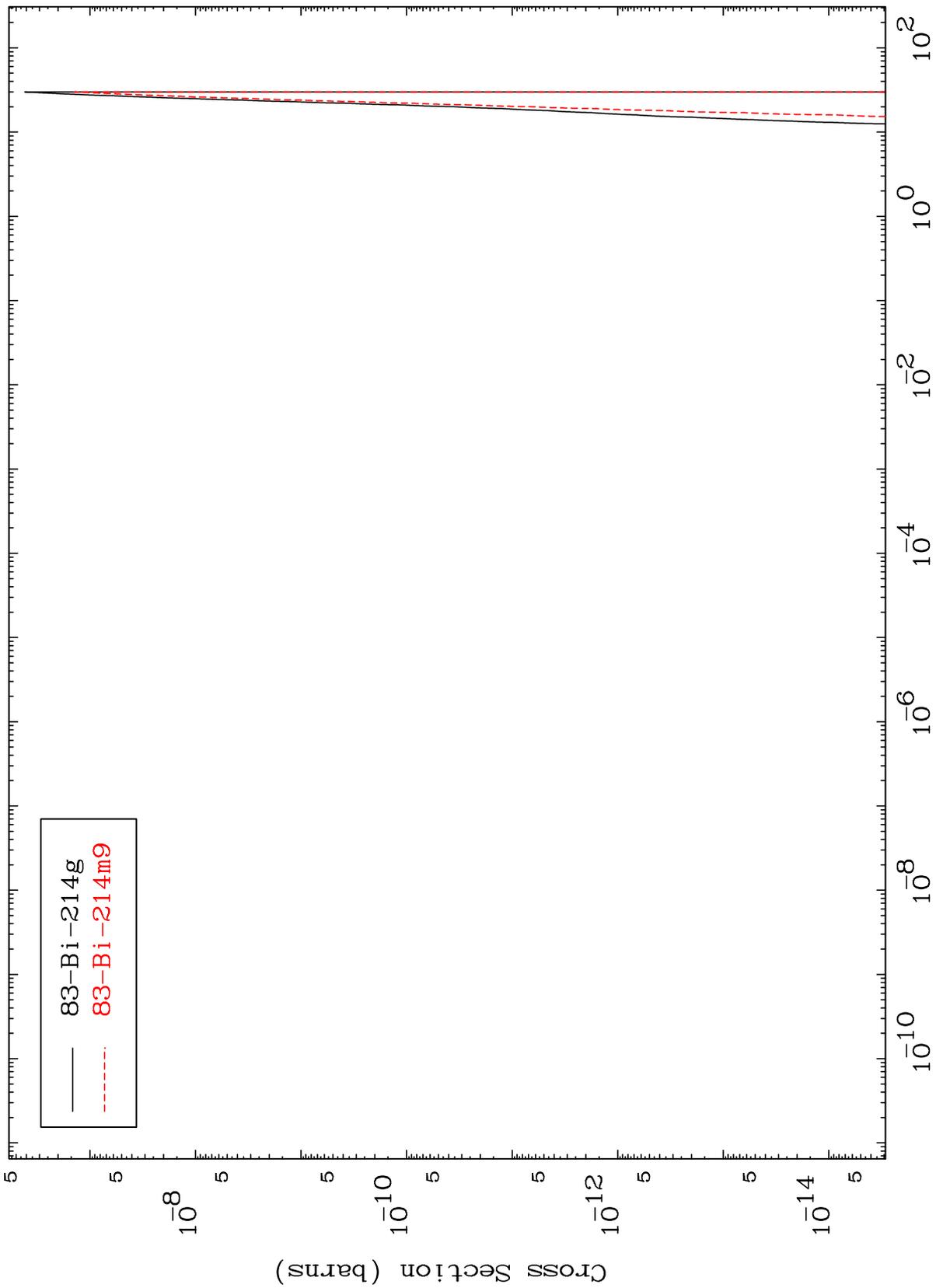


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(n,n') p α

86-Rn-219

Radionuclide Production Cross Section



83-Bi-214g
83-Bi-214m9

30

Incident Energy (MeV)

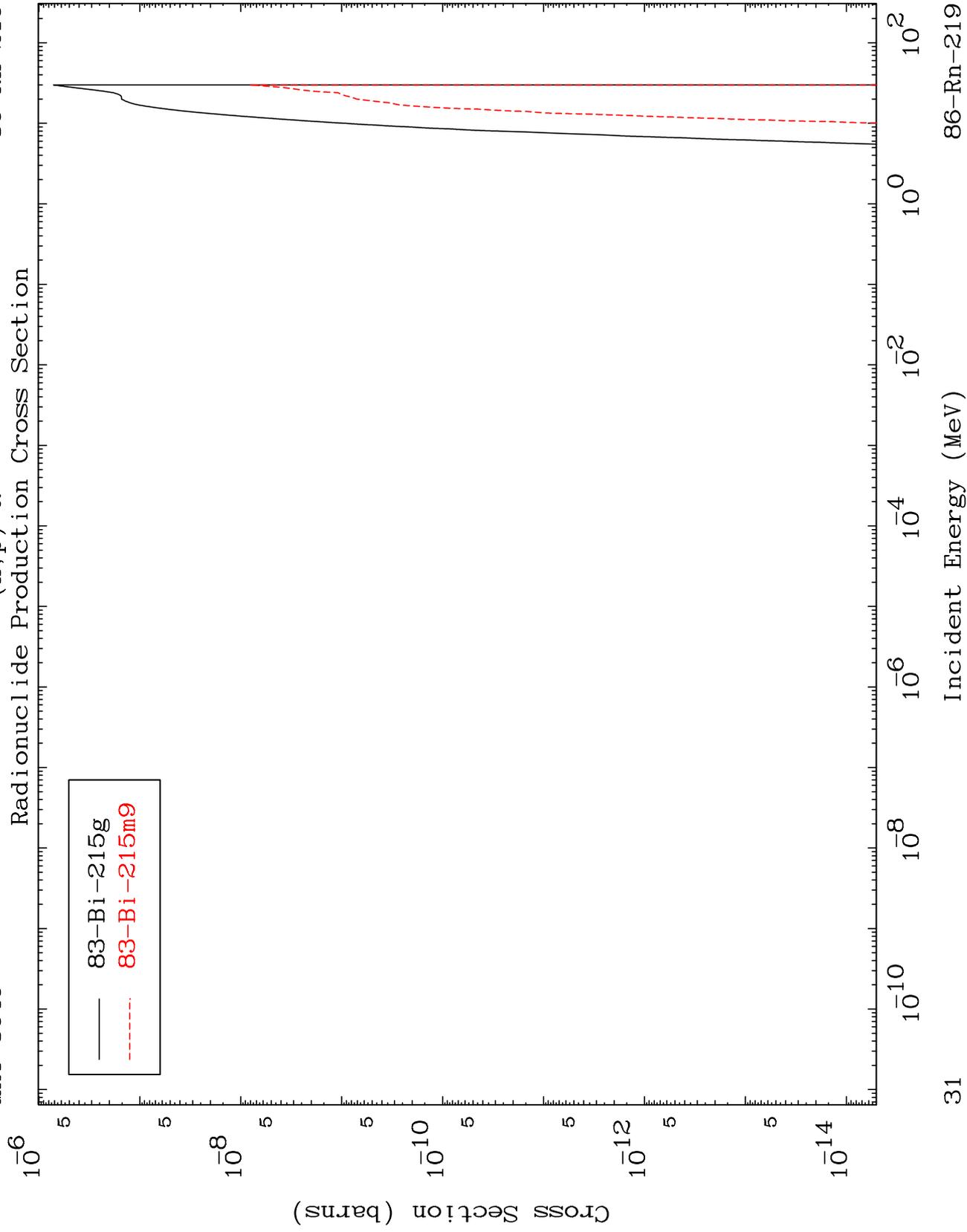
86-Rn-219

MAT 8649

(n,p) α

86-Rn-219

Radionuclide Production Cross Section

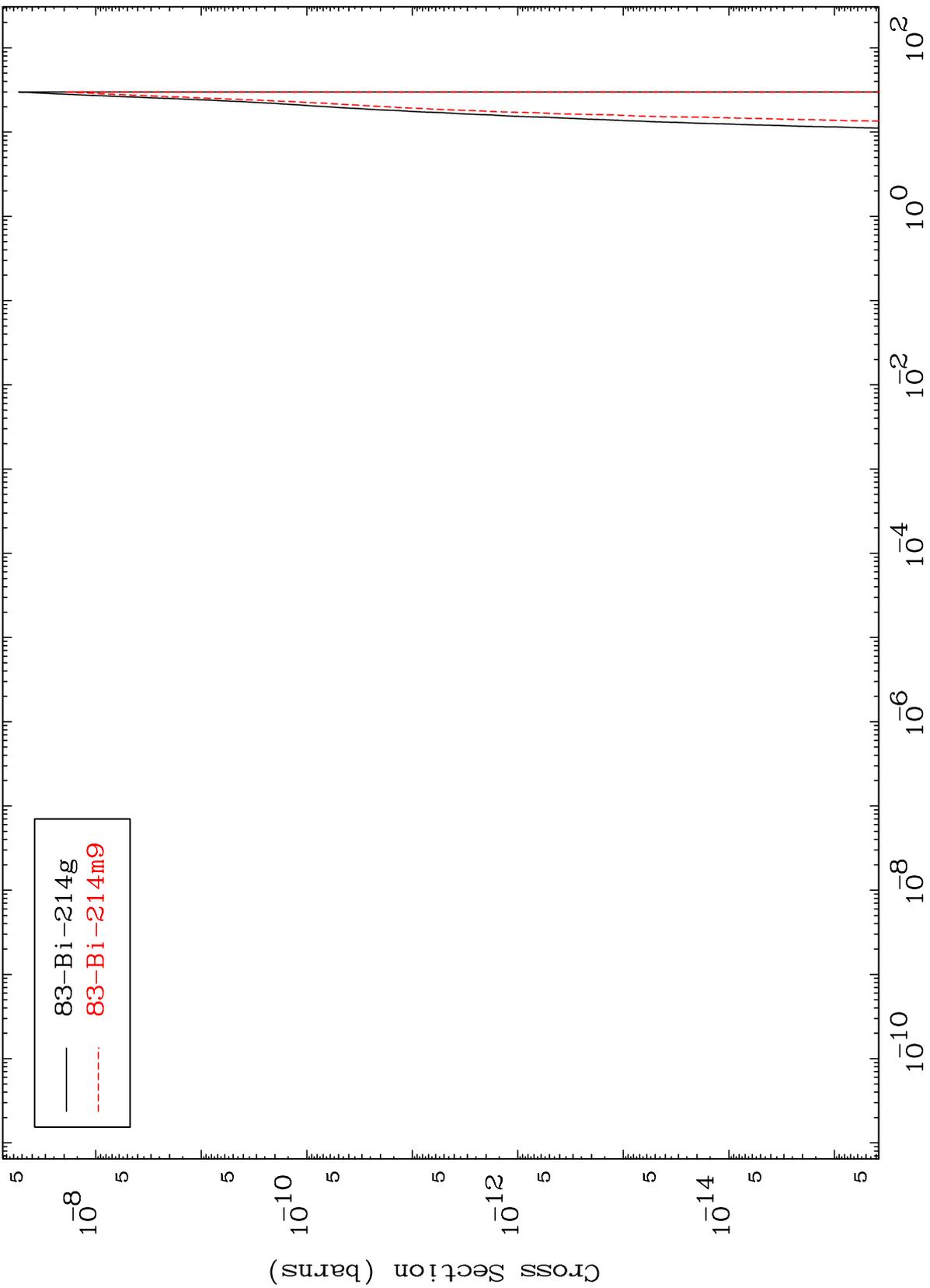


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(n,d) α

86-Rn-219

Radionuclide Production Cross Section



83-Bi-214g
83-Bi-214m9

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Incident Energy (MeV)

86-Rn-219