

Program EVALPLOT  
(Version 2021-1)

by

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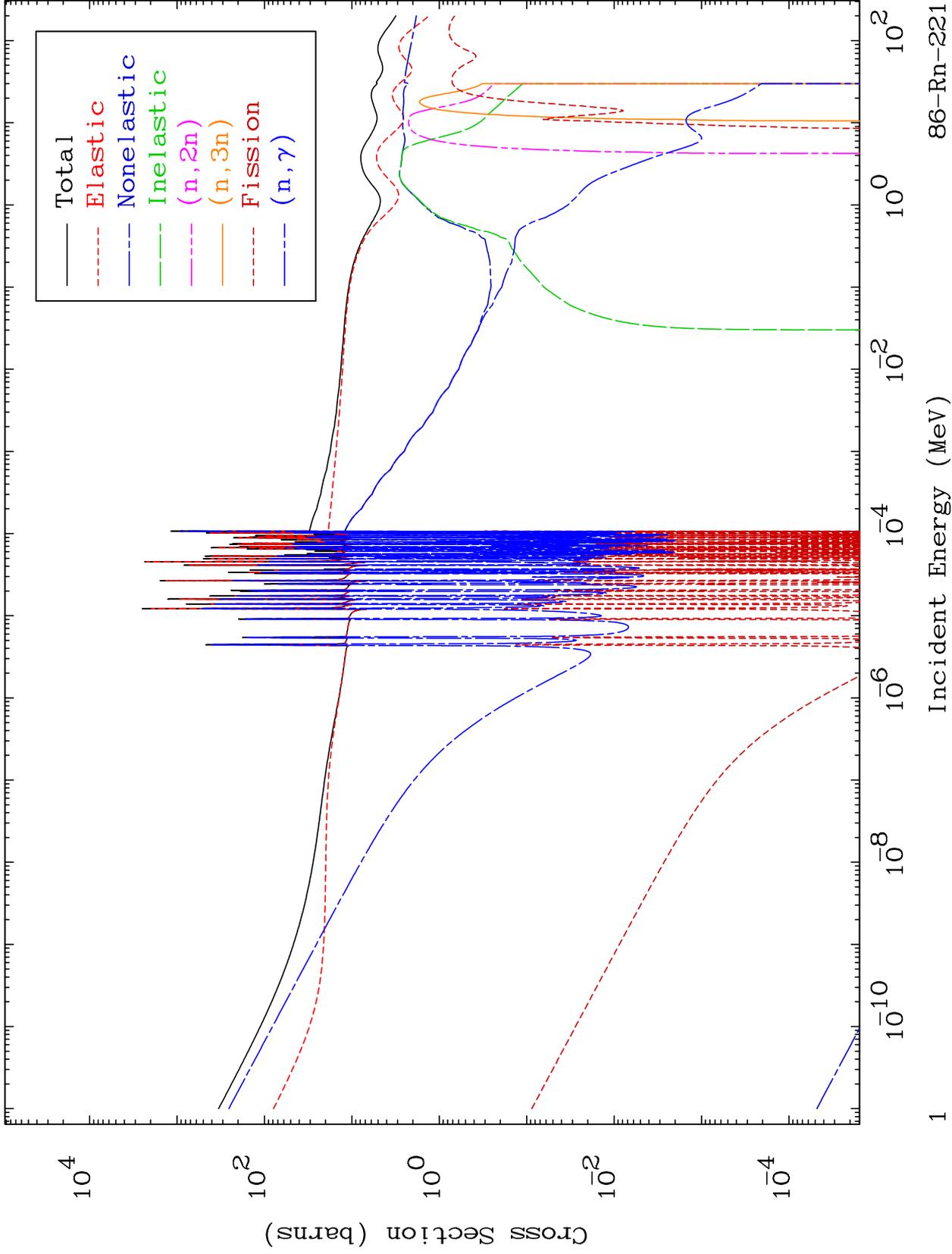
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Press Mouse Button to Start

MAT 8655

Neutron Major  
293 Kelvin Cross Sections

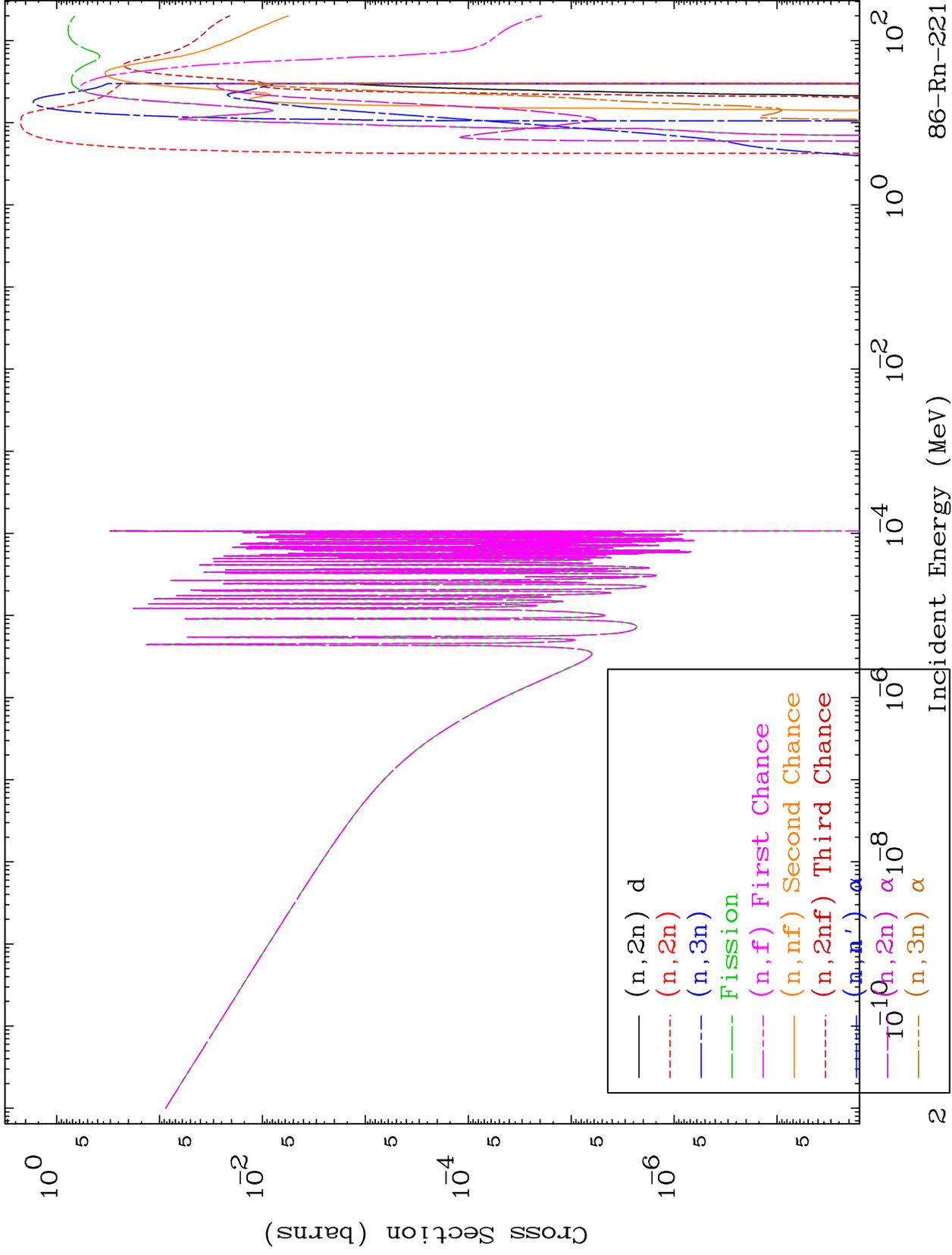
86-Rn-221



MAT 8655

Neutron Absorption  
293 Kelvin Cross Sections

86-Rn-221

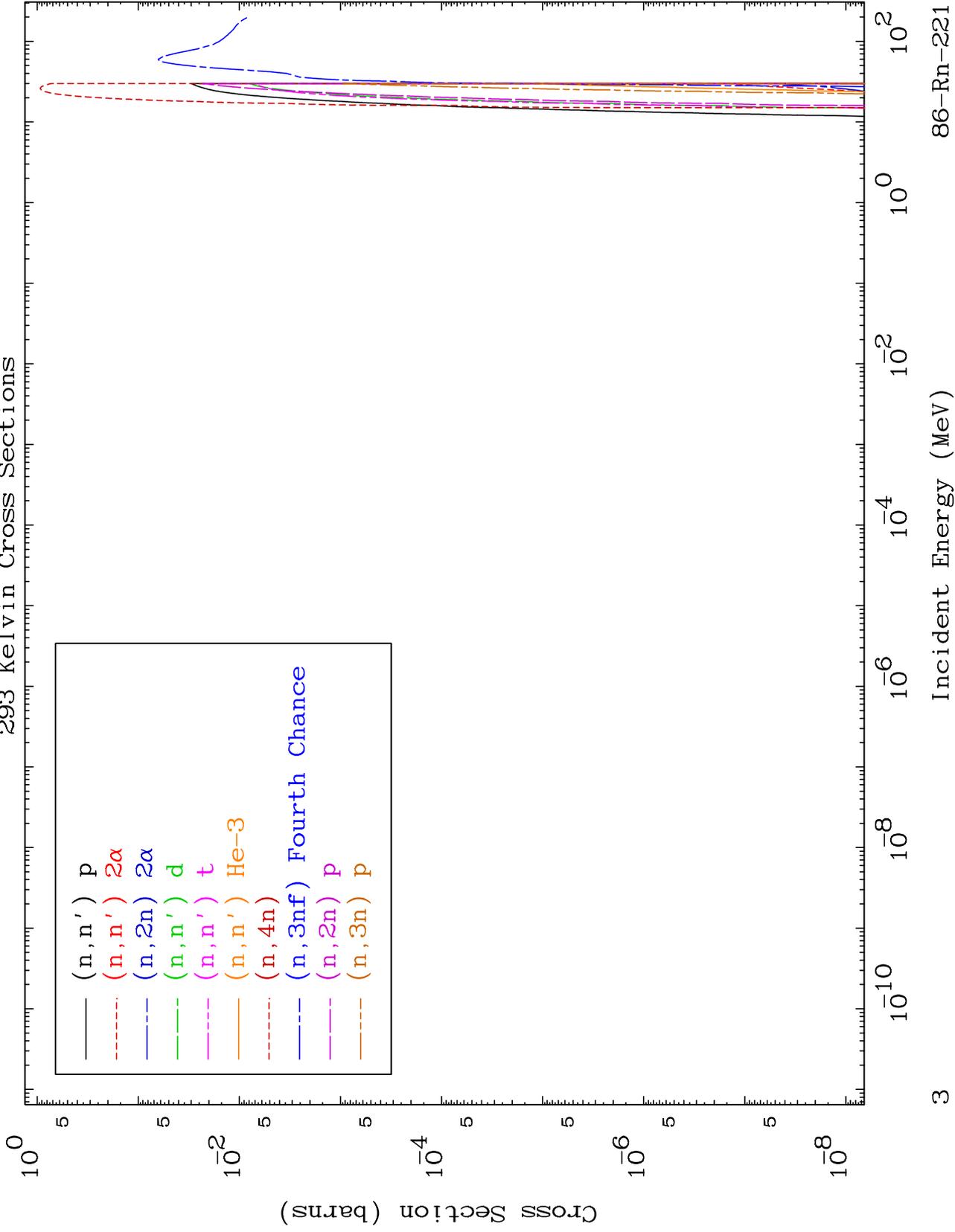


86-Rn-221

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Neutron Absorption  
293 Kelvin Cross Sections

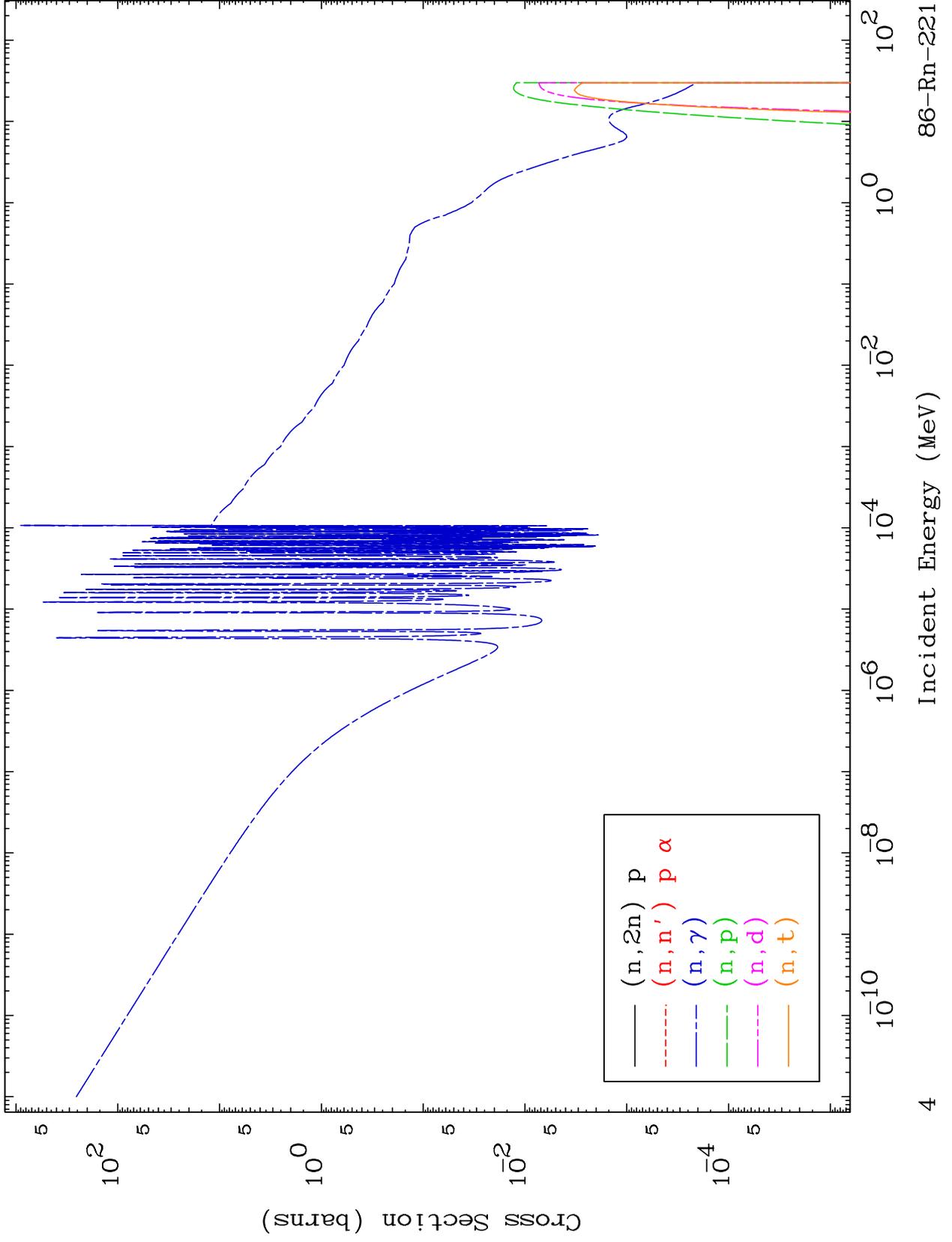
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Neutron Absorption  
293 Kelvin Cross Sections

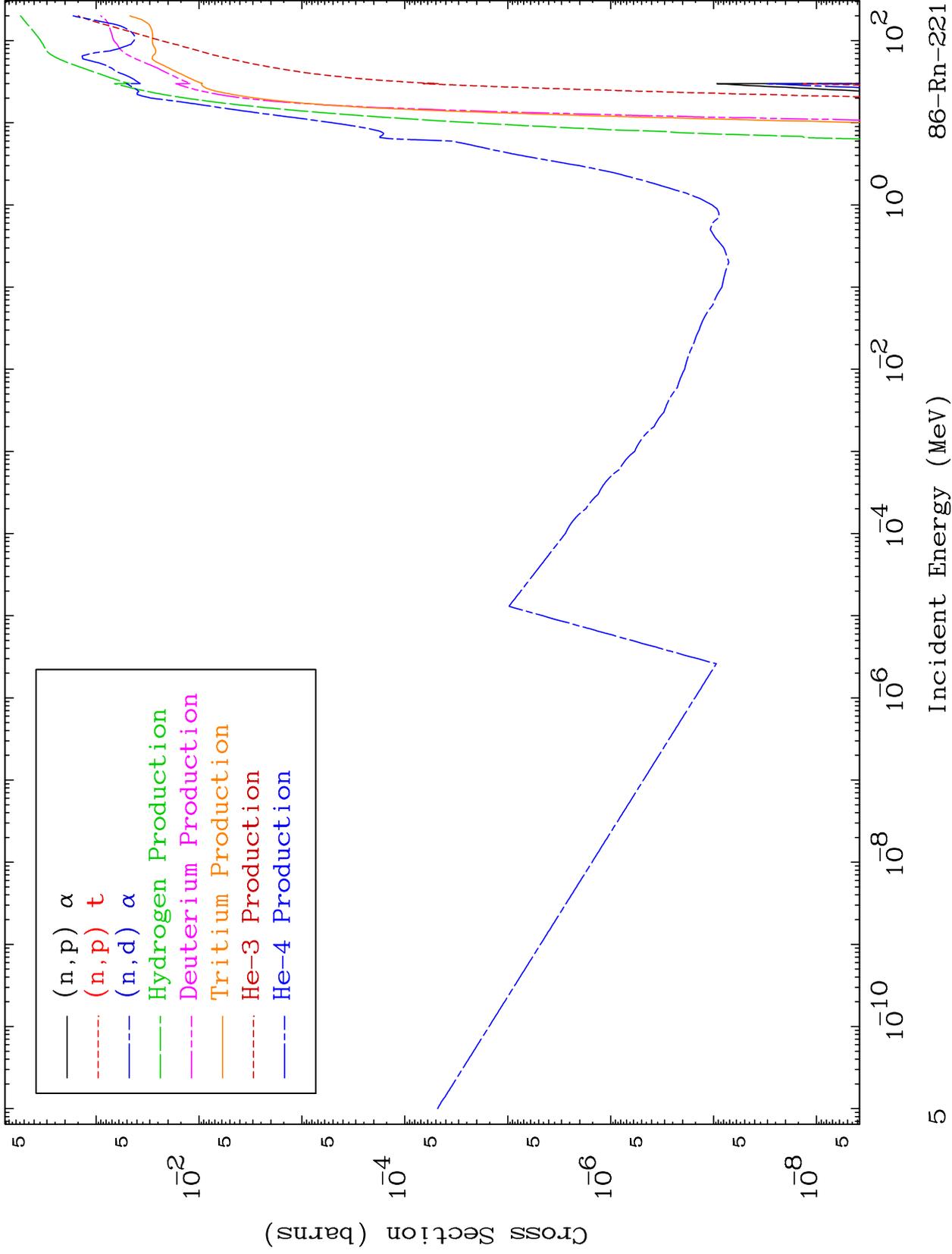
86-Rn-221



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Neutron Absorption  
293 Kelvin Cross Sections

86-Rn-221

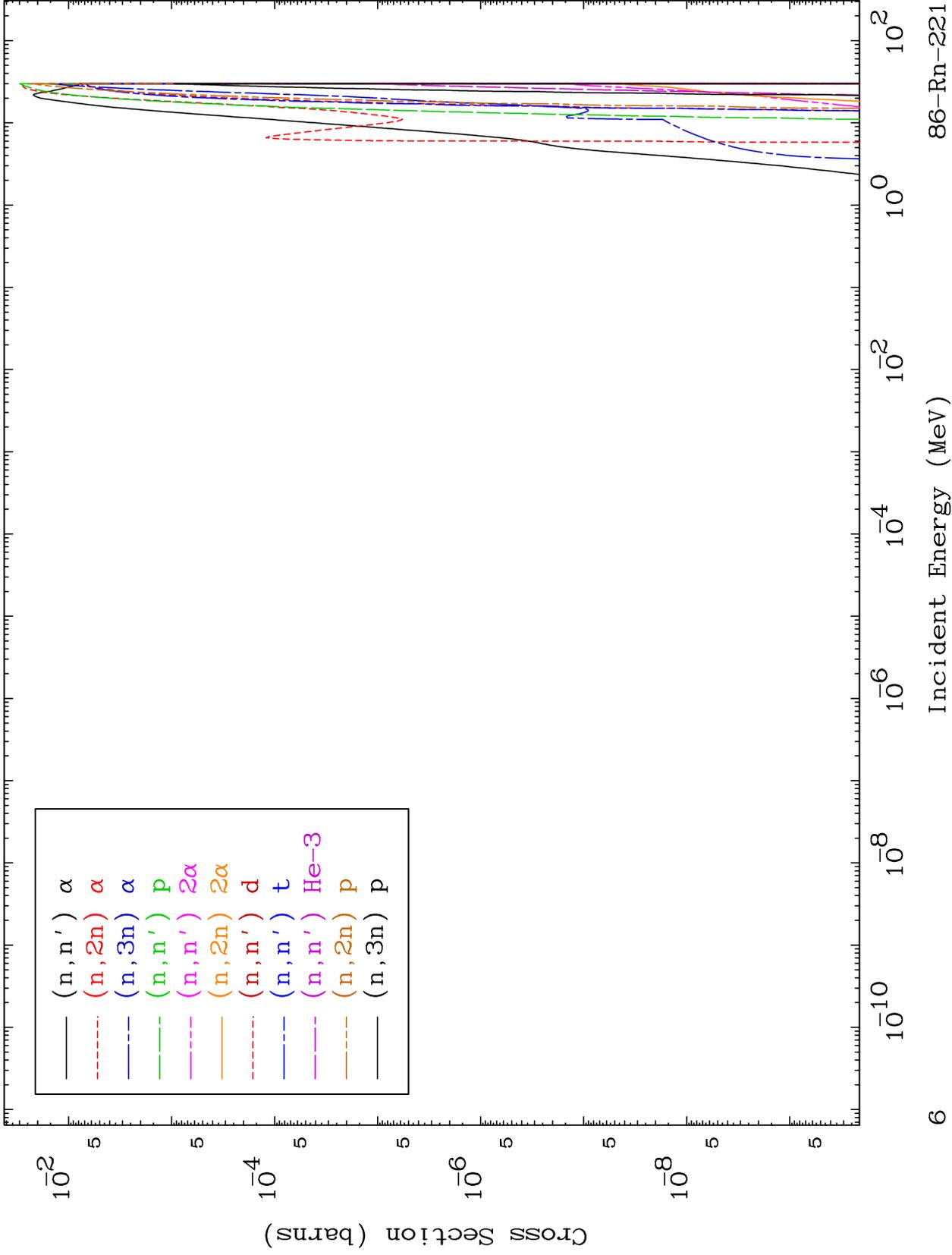


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Charged Particle  
293 Kelvin Cross Sections

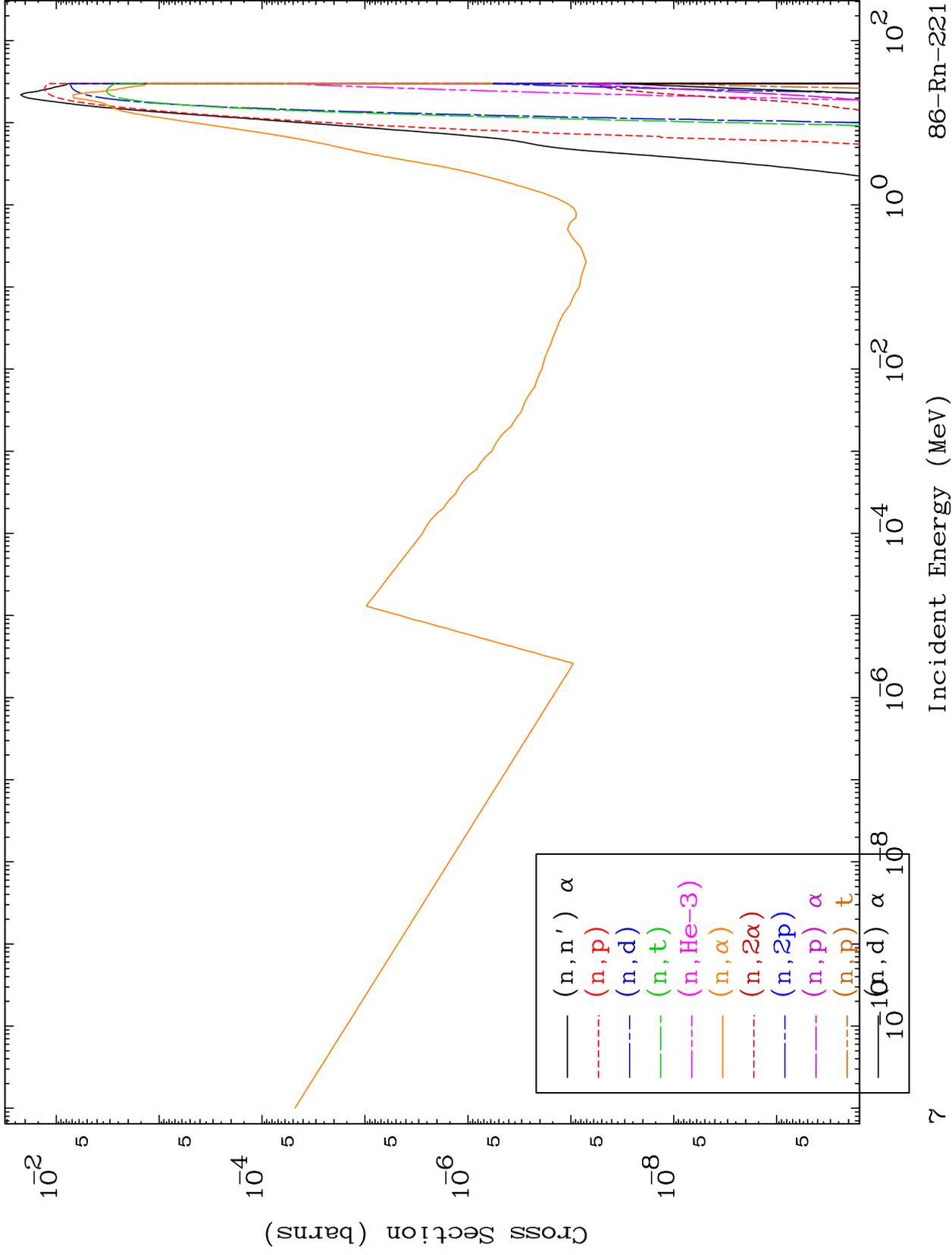
86-Rn-221



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Charged Particle  
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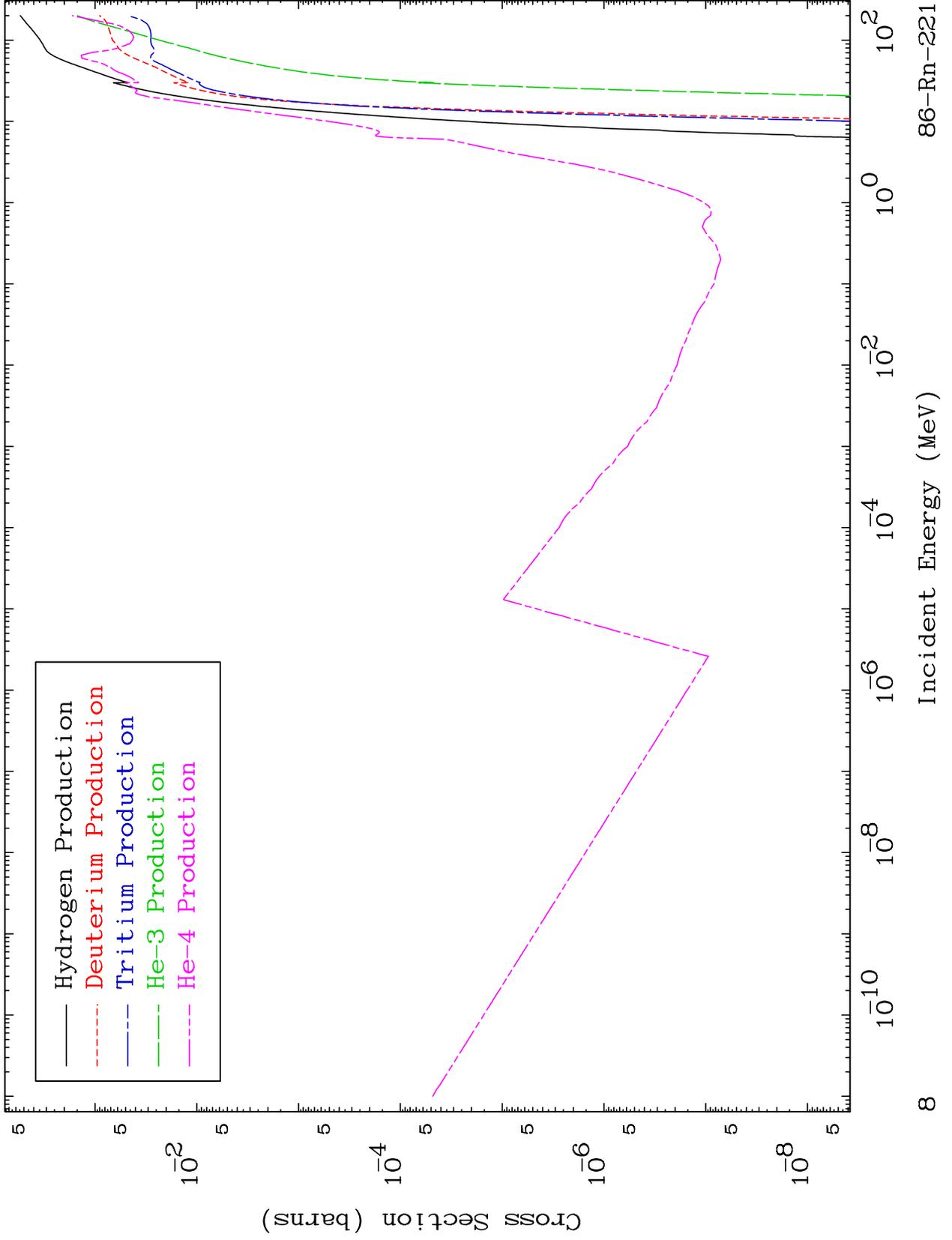
86-Rn-221



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Particle Production  
293 Kelvin Cross Sections

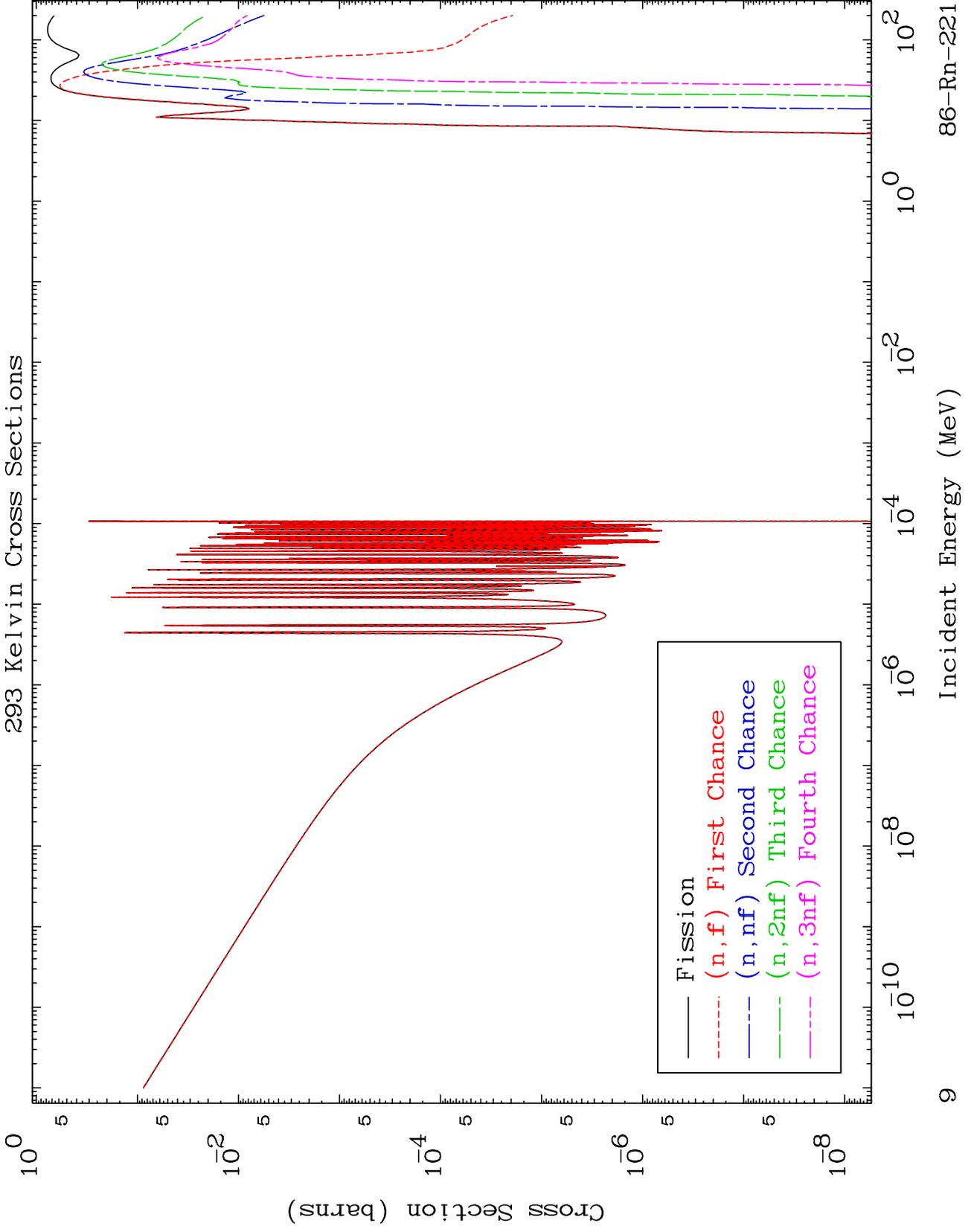
86-Rn-221



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Fission  
293 Kelvin Cross Sections

86-Rn-221

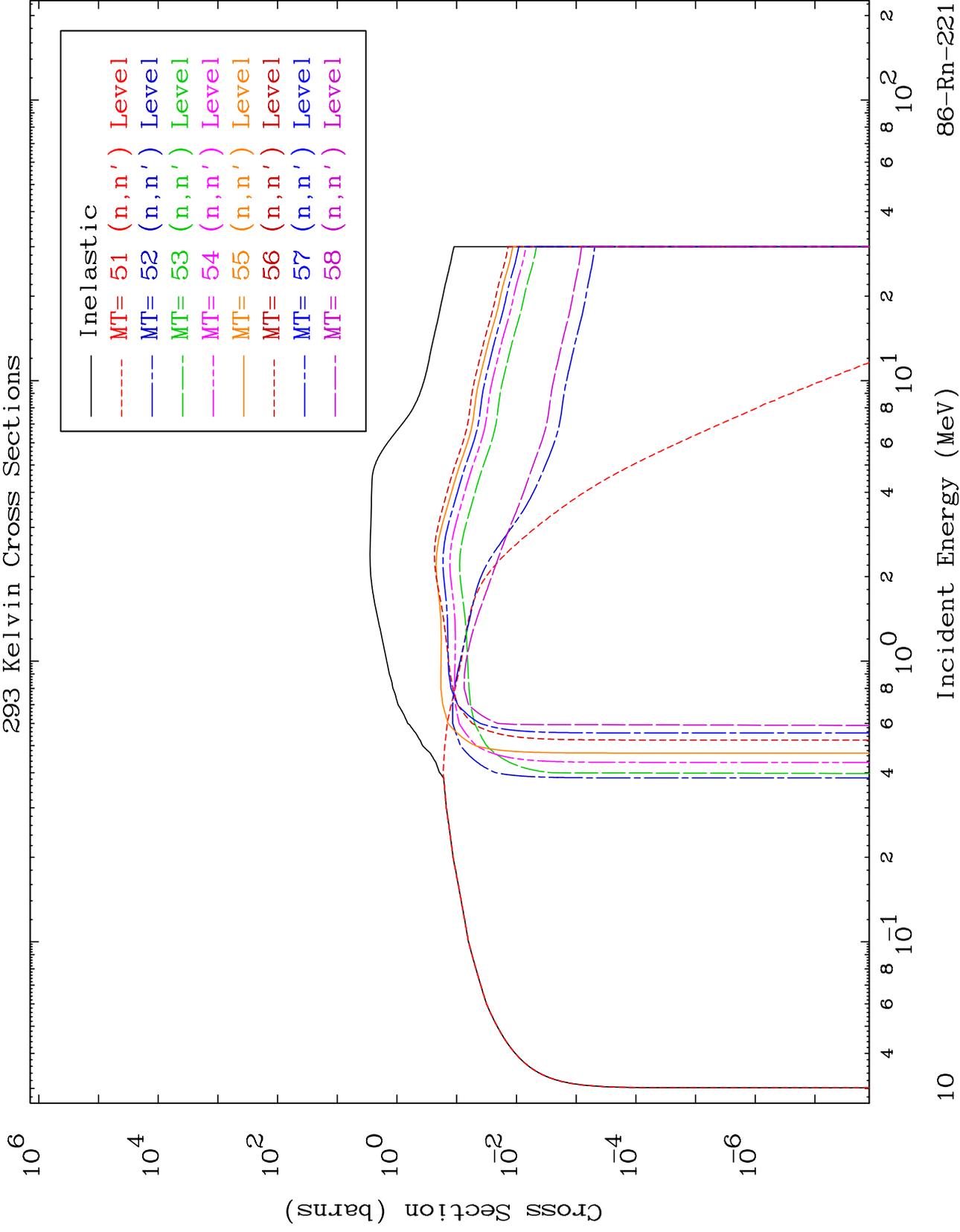


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(n,n') Levels

293 Kelvin Cross Sections

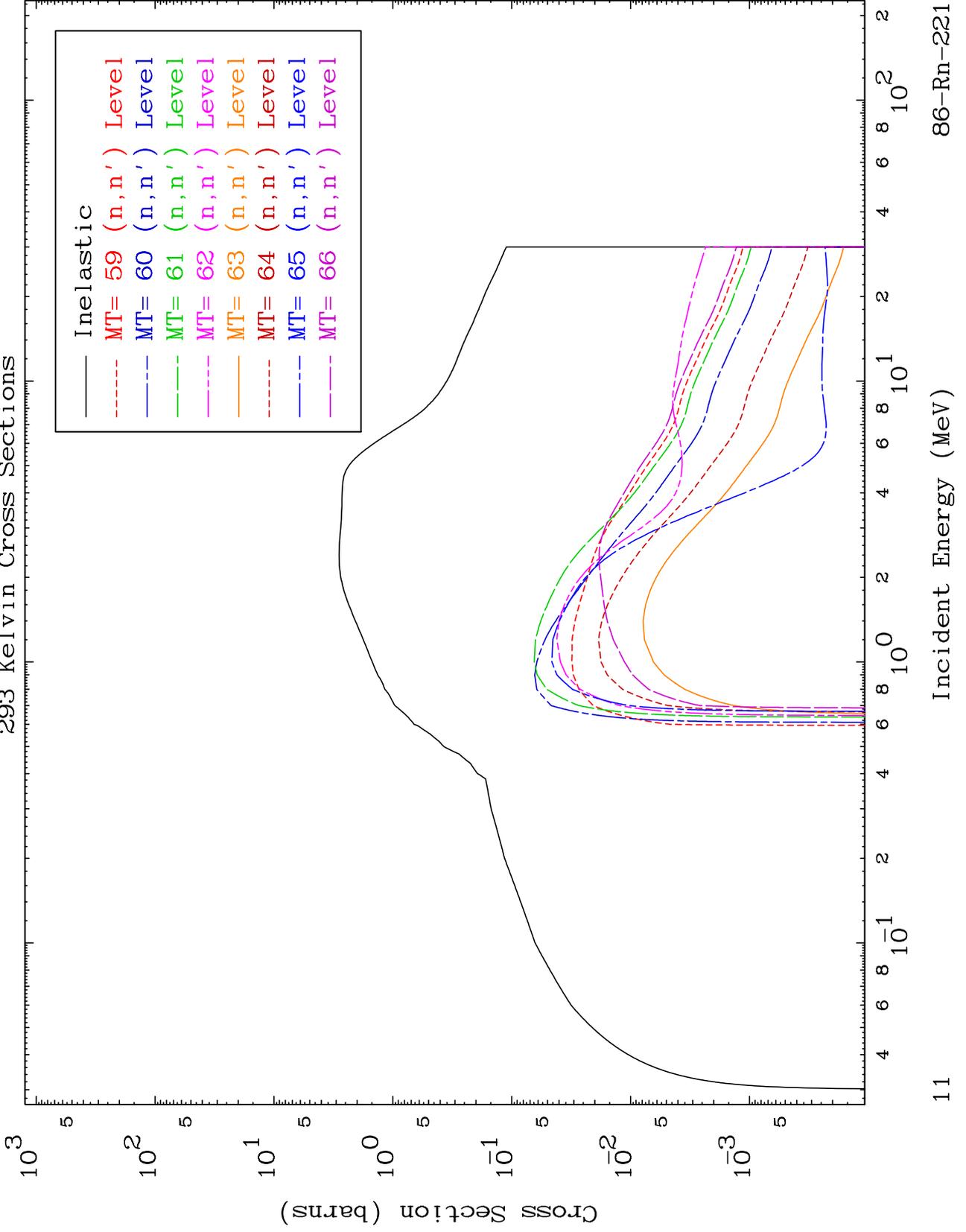
86-Rn-221



10

Incident Energy (MeV)

86-Rn-221

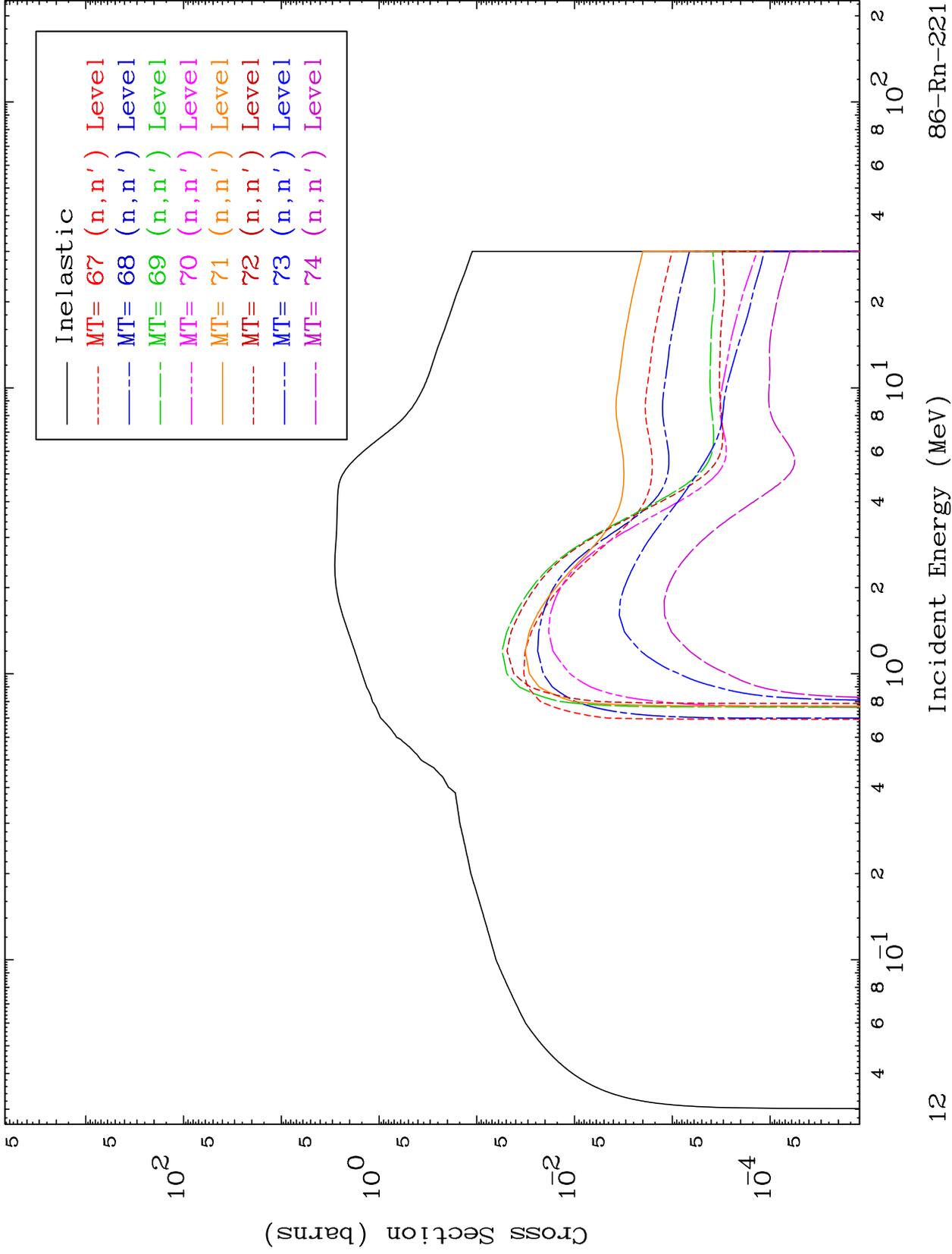


MAT 8655

(n,n') Levels

86-Rn-221

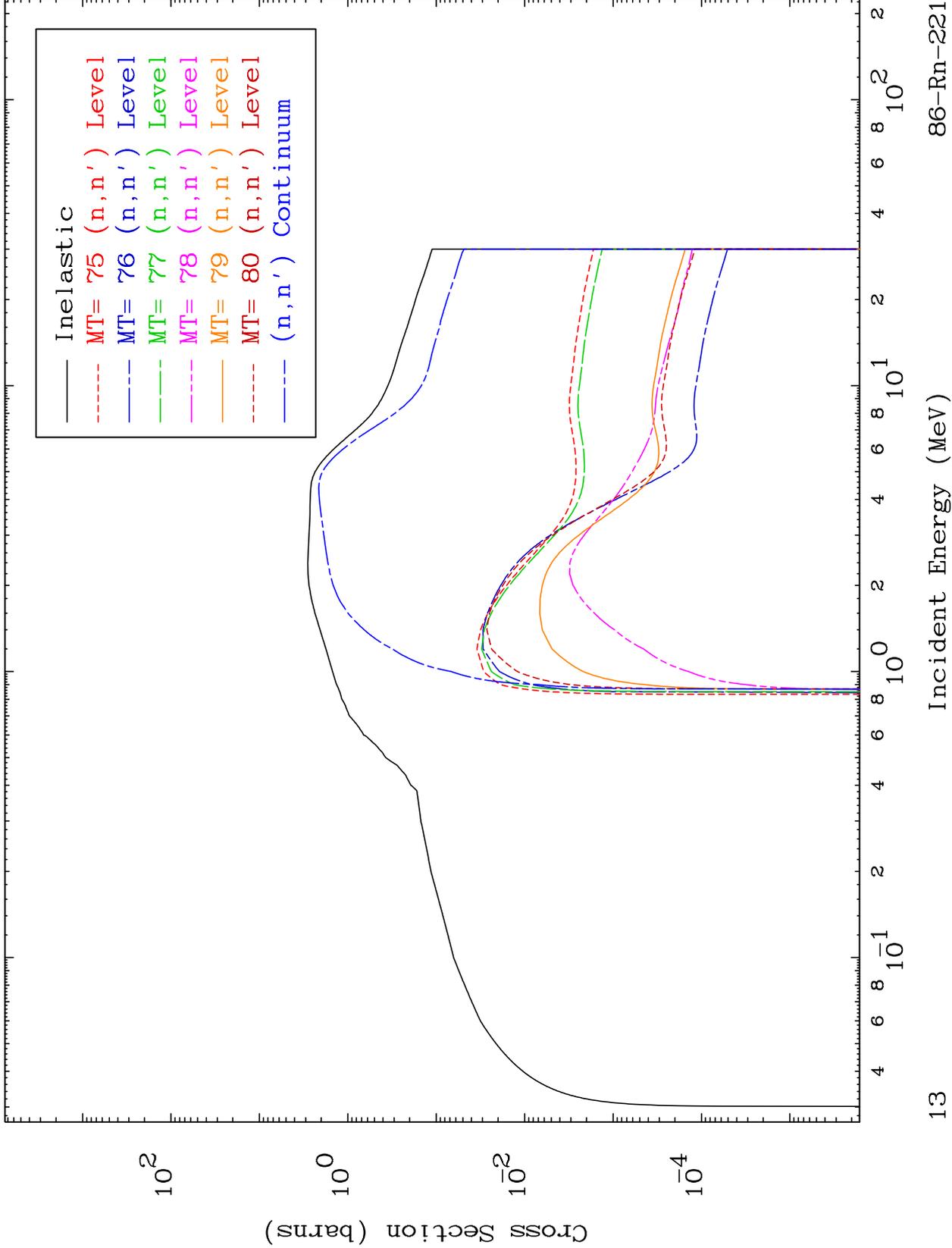
293 Kelvin Cross Sections



12

Incident Energy (MeV)

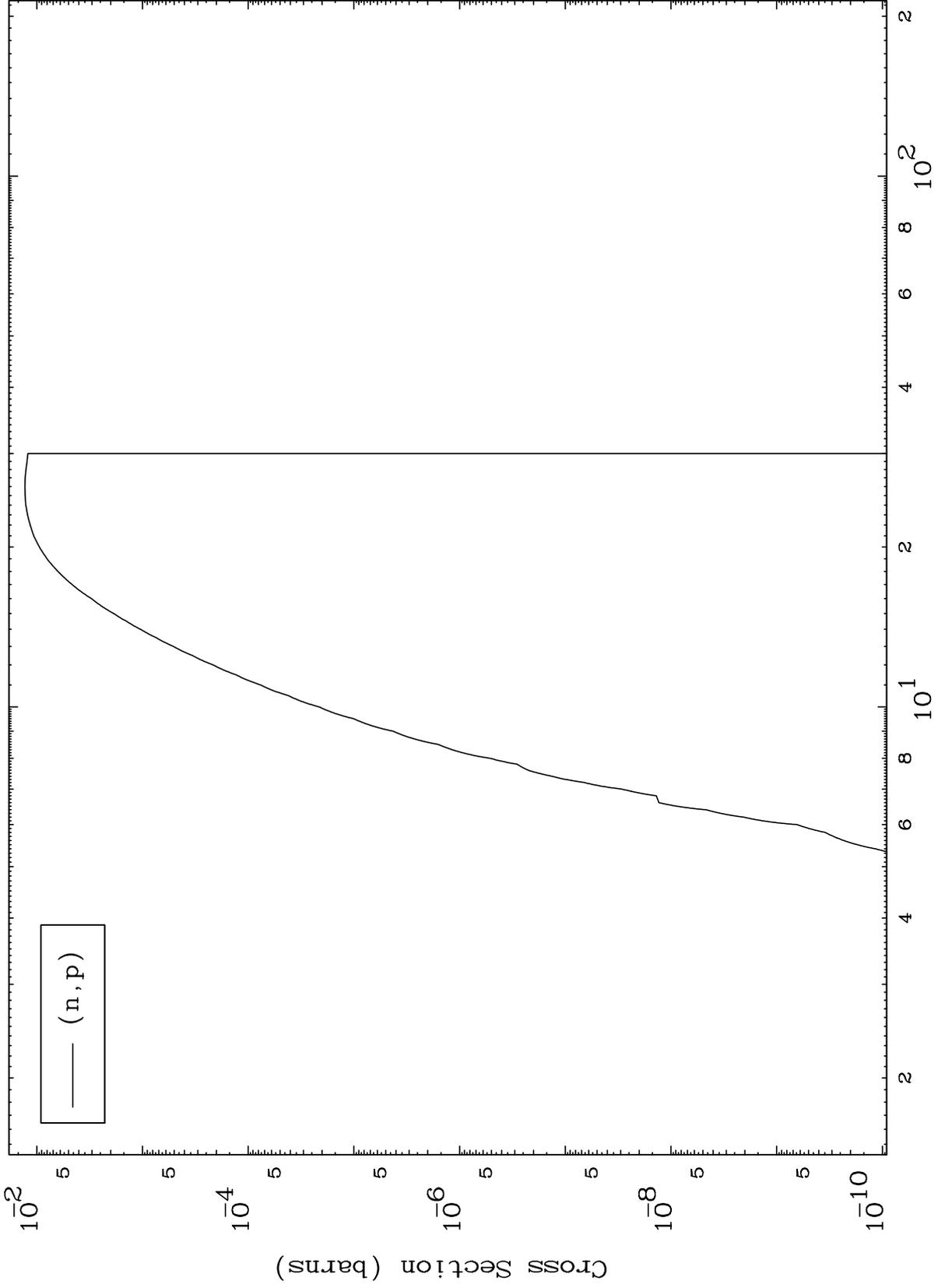
86-Rn-221



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(n,p) Levels  
293 Kelvin Cross Sections

86-Rn-221



14

Incident Energy (MeV)

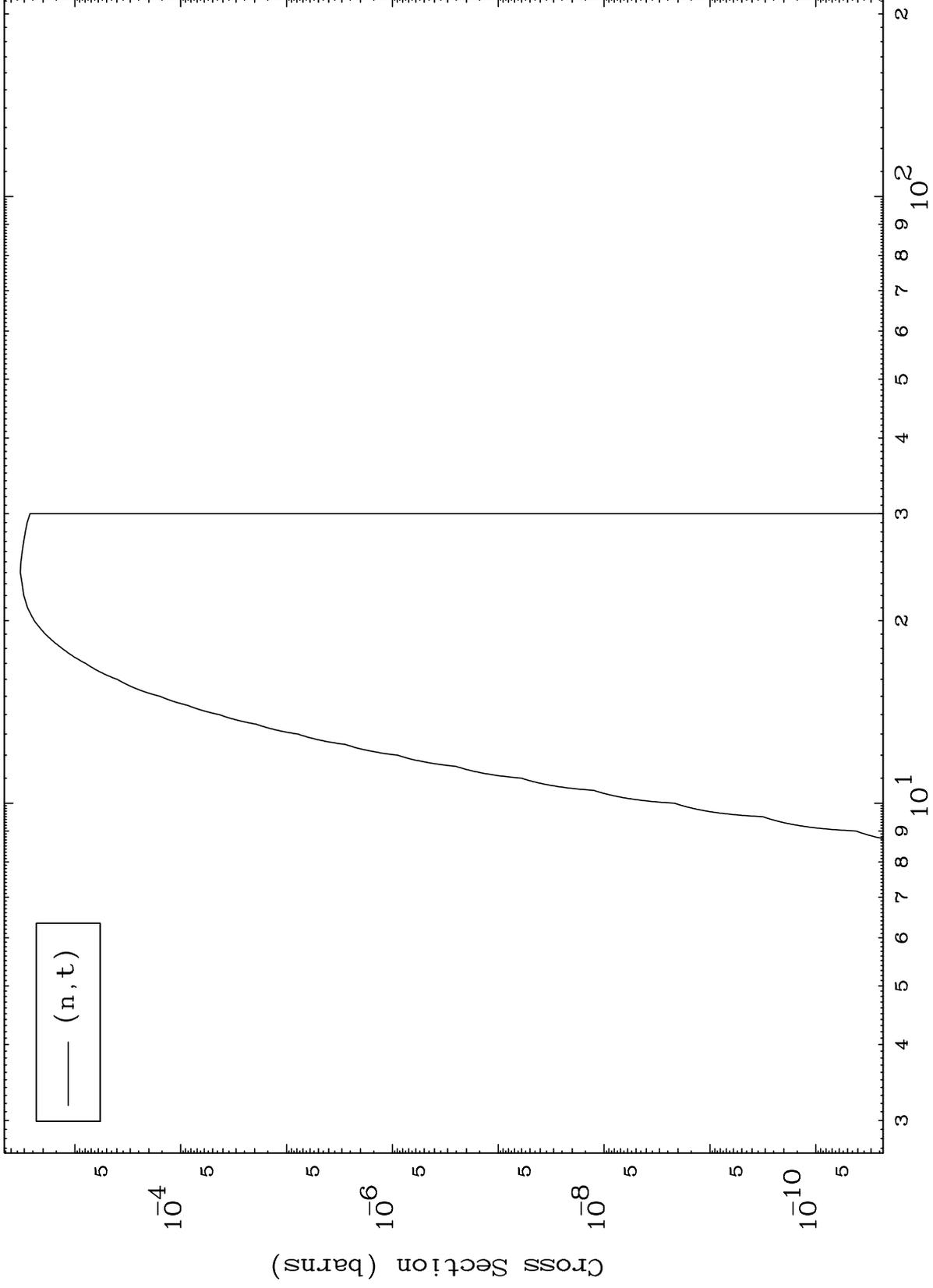
86-Rn-221



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(n,t) Levels  
293 Kelvin Cross Sections

86-Rn-221



16

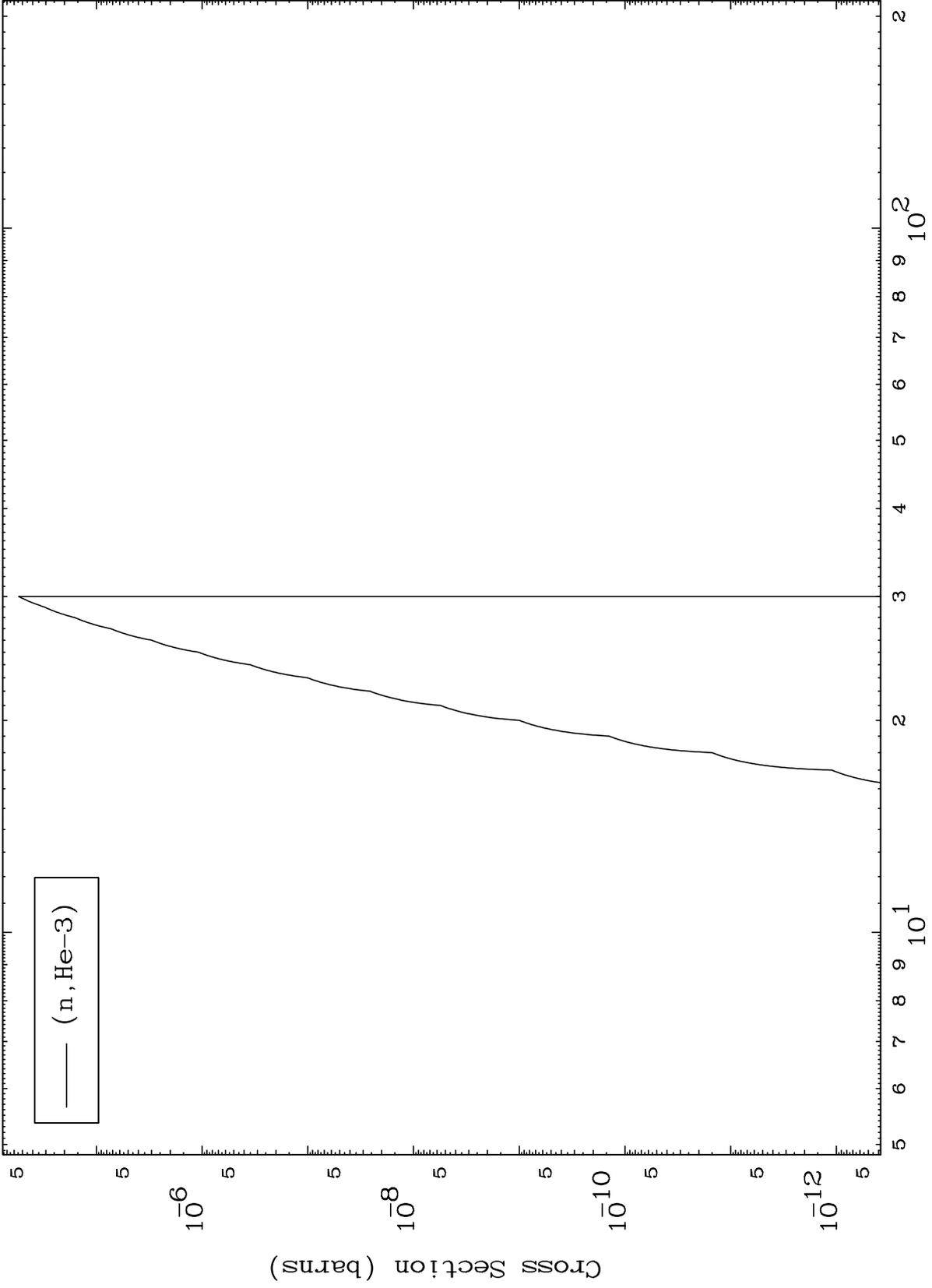
Incident Energy (MeV)

86-Rn-221

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(n,He3) Levels  
293 Kelvin Cross Sections

86-Rn-221



17

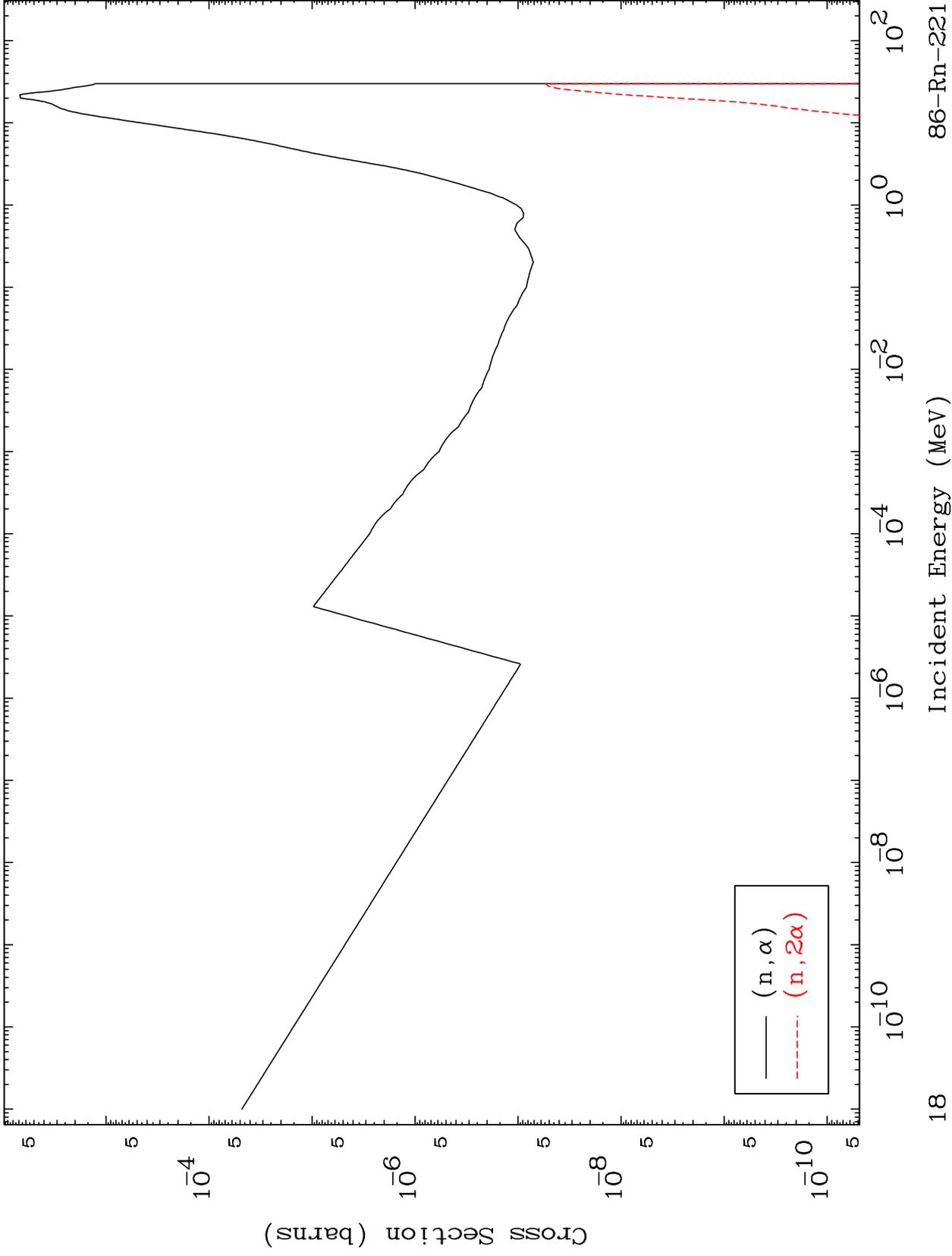
Incident Energy (MeV)

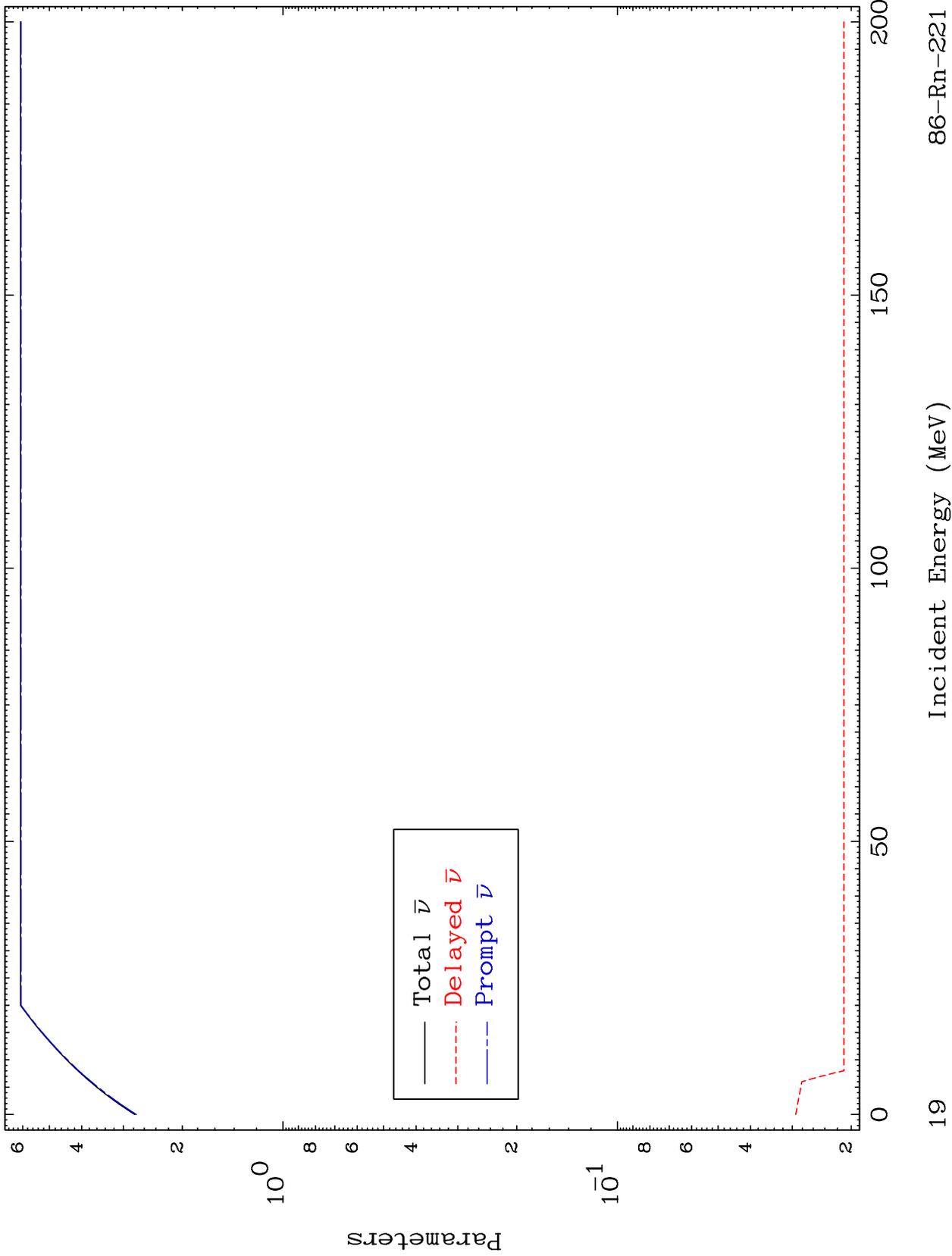
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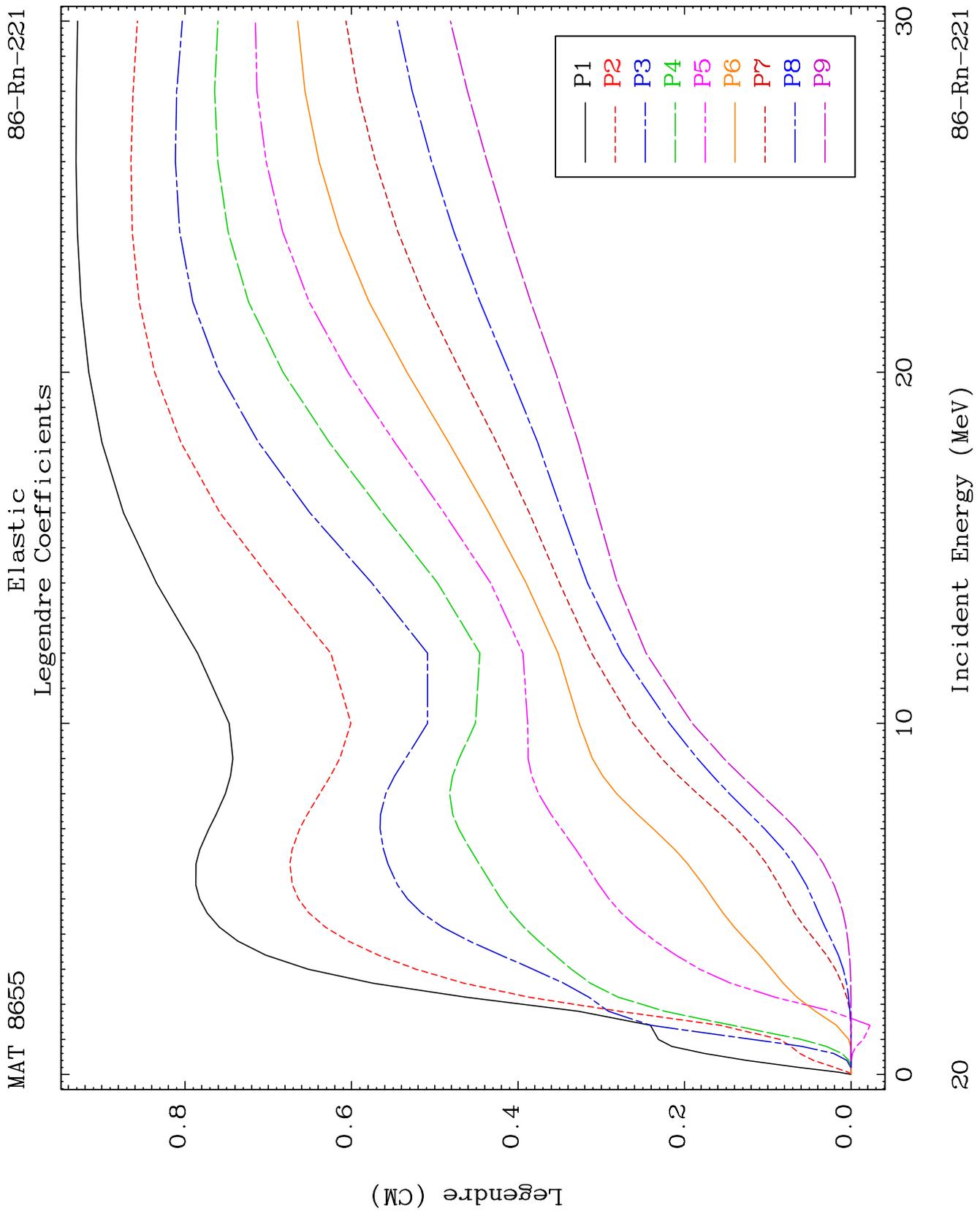
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(n,α) Levels  
293 Kelvin Cross Sections

86-Rn-221



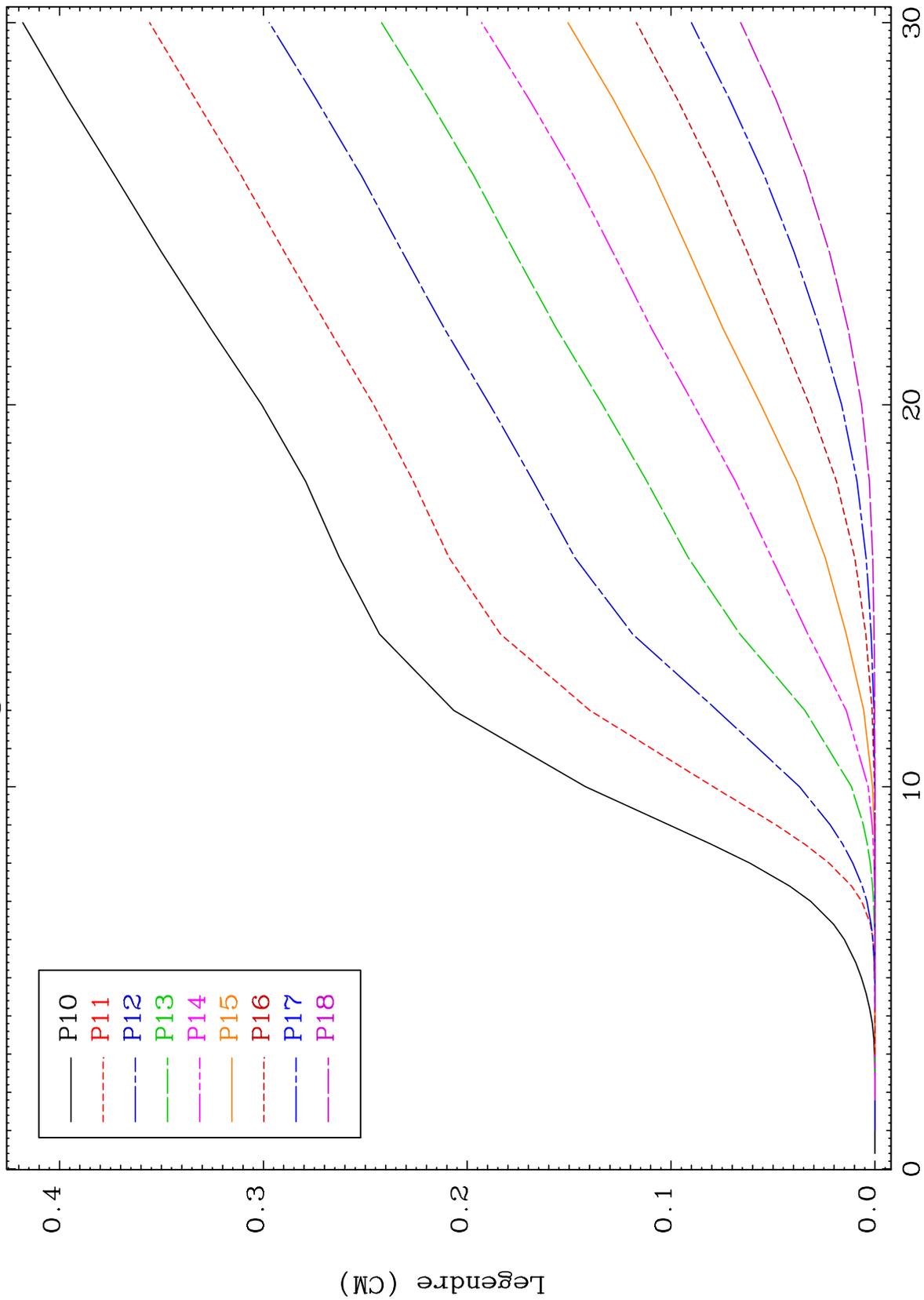




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Elastic Legendre Coefficients

86-Rn-221



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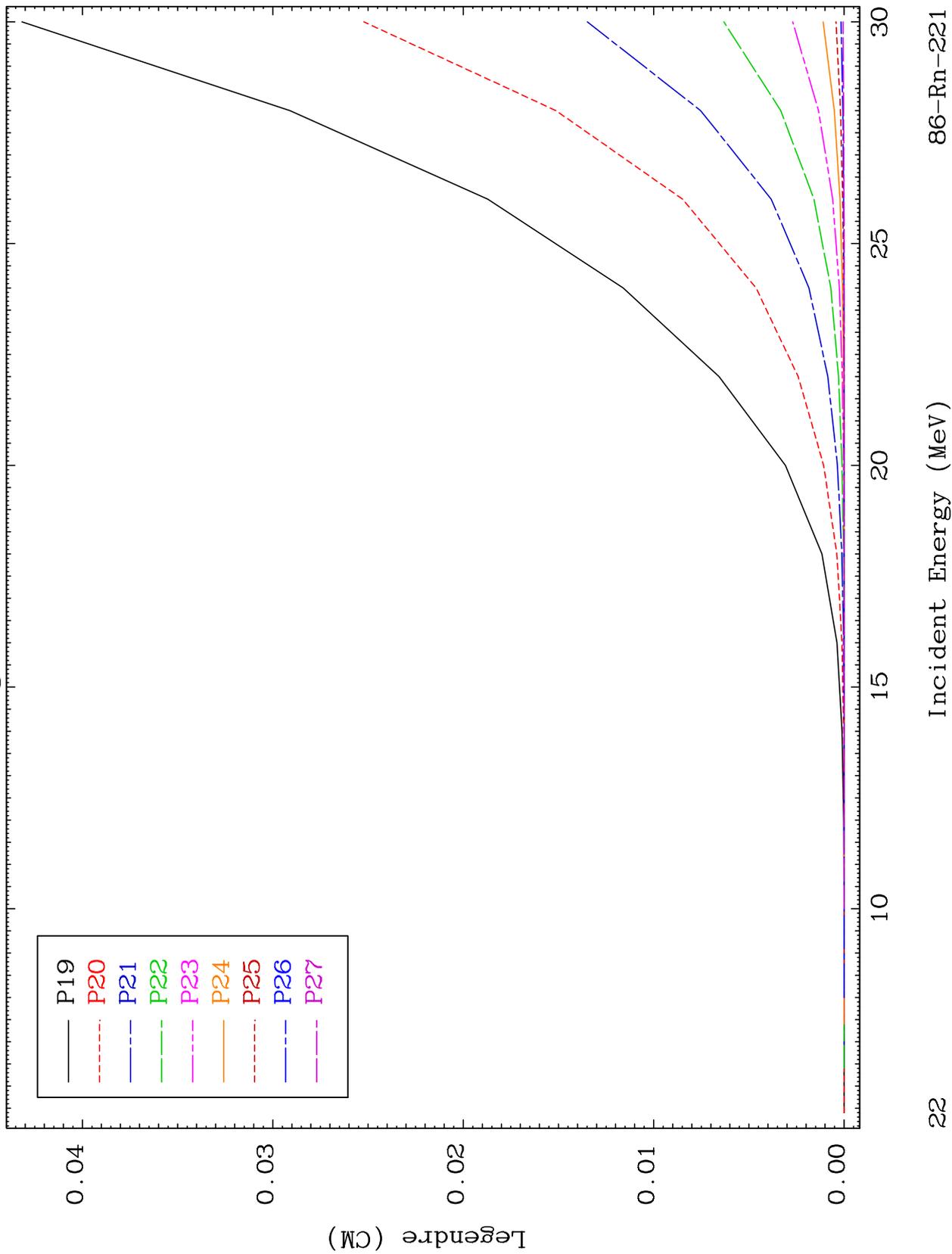
Incident Energy (MeV)

86-Rn-221

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Elastic Legendre Coefficients

86-Rn-221



22

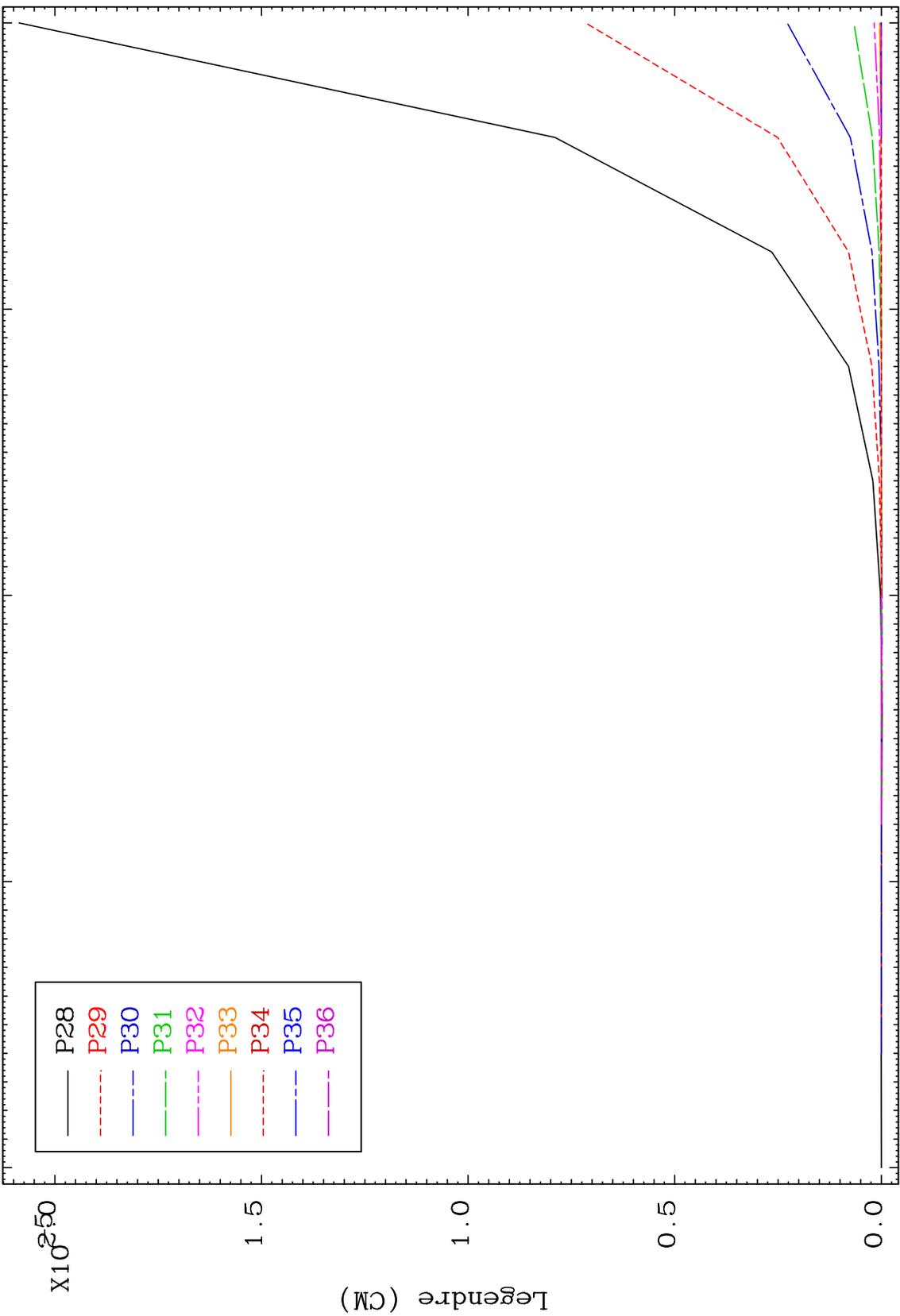
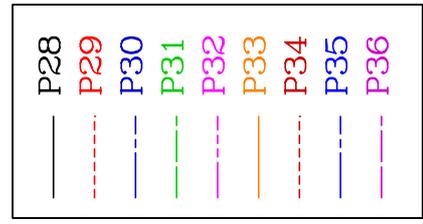
Incident Energy (MeV)

86-Rn-221

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Elastic Legendre Coefficients

86-Rn-221



86-Rn-221

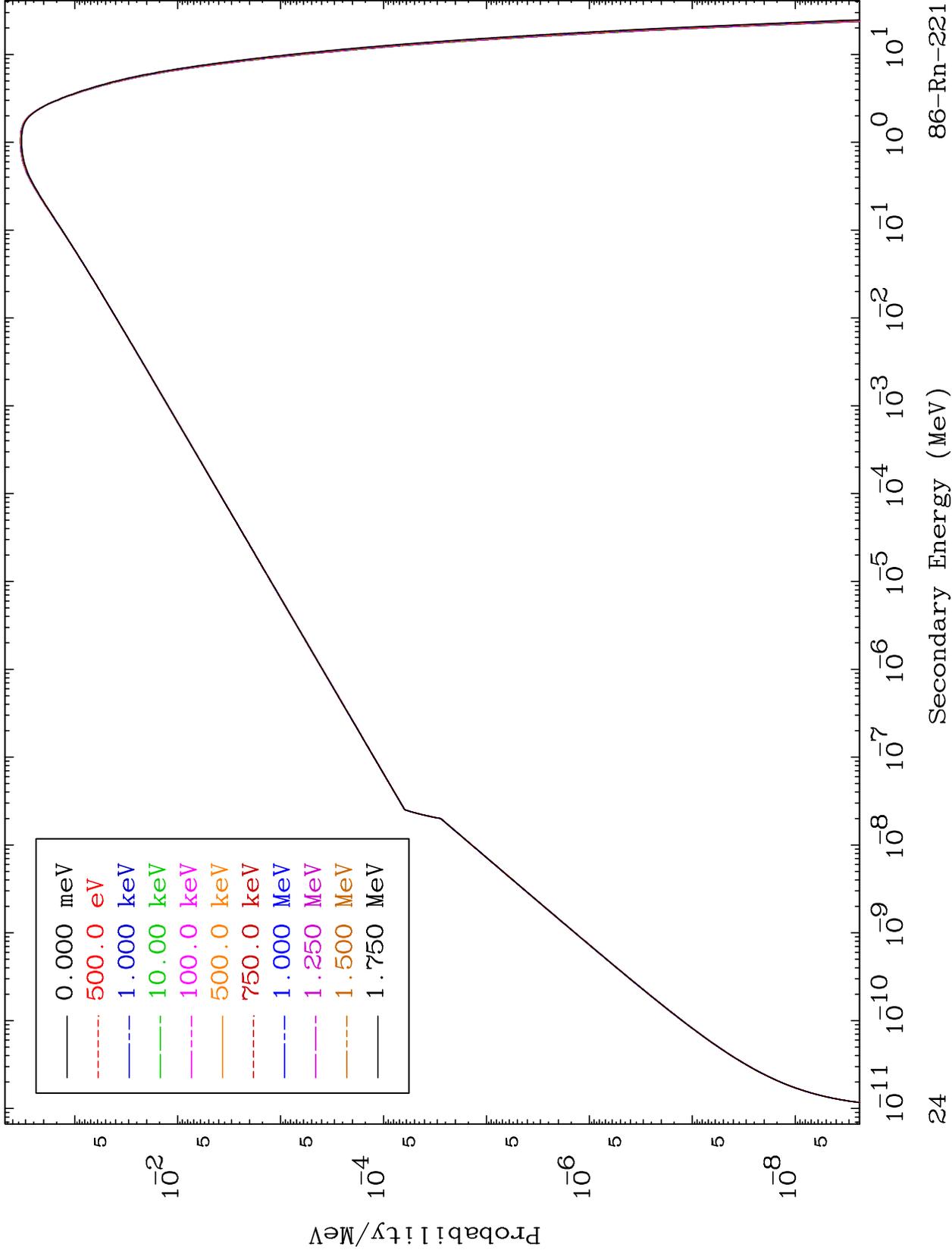
Incident Energy (MeV)

23

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### Fission Energy Distributions

86-Rn-221



86-Rn-221

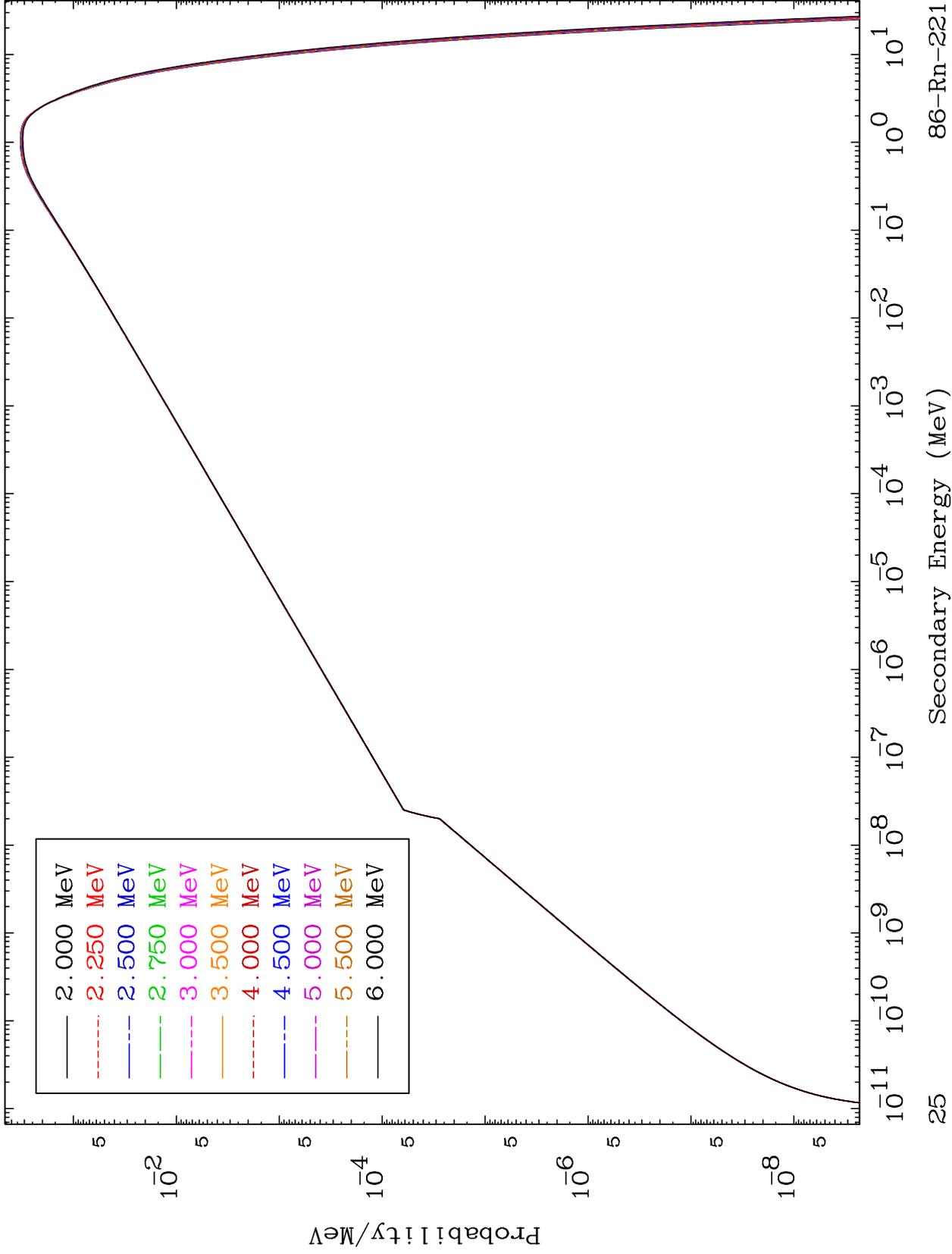
Secondary Energy (MeV)

24

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### Fission Energy Distributions

86-Rn-221



86-Rn-221

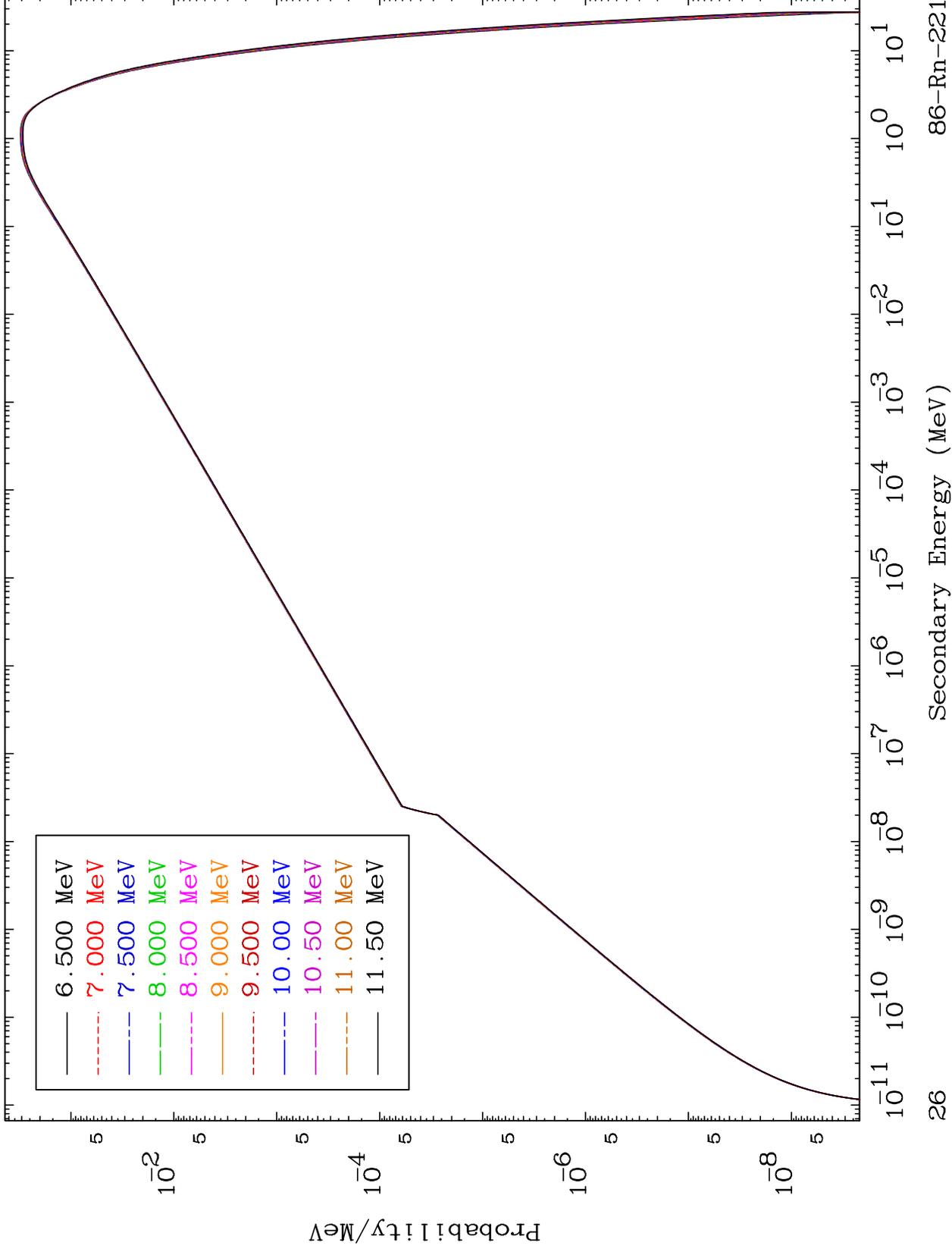
Secondary Energy (MeV)

25

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### Fission Energy Distributions

86-Rn-221



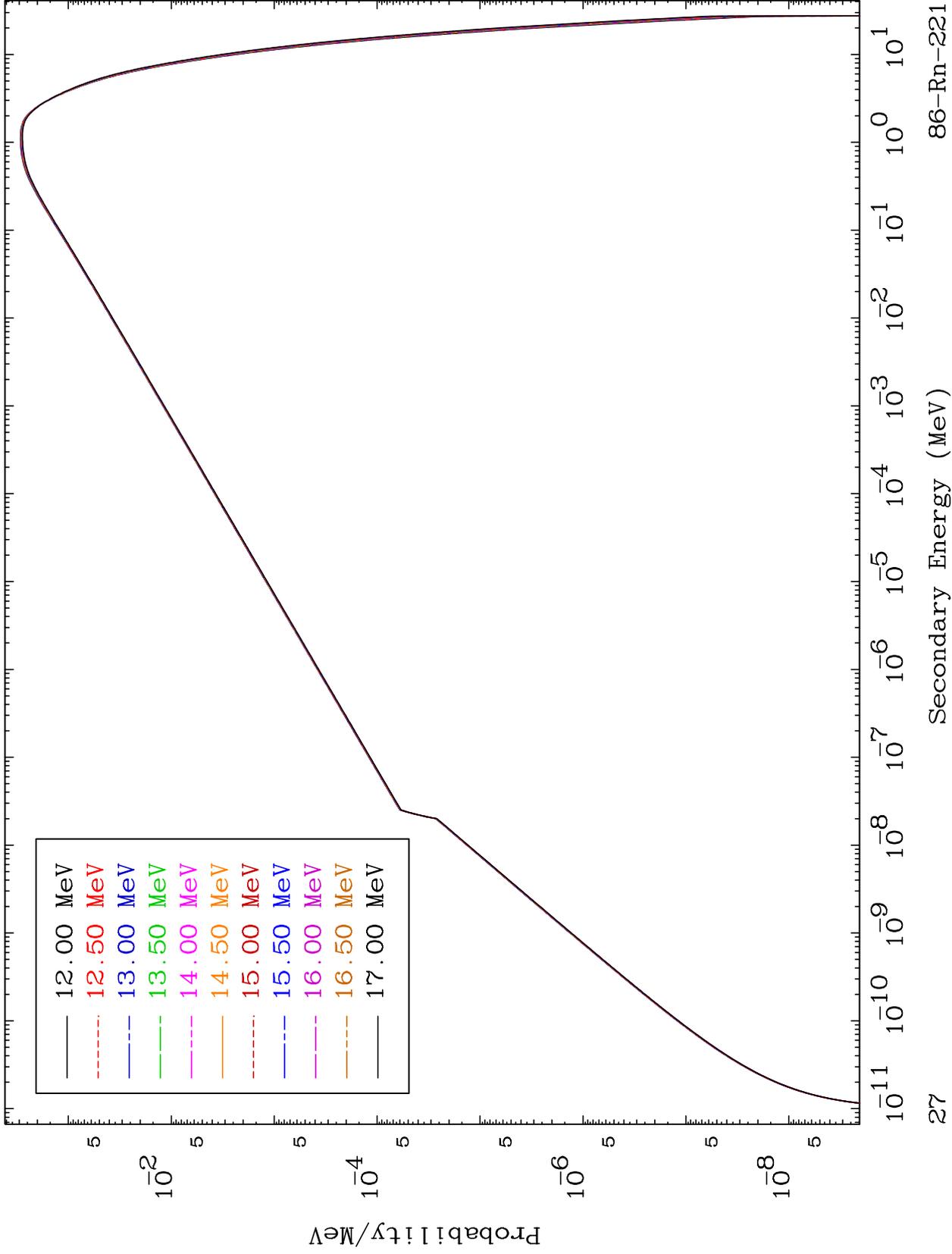
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86-Rn-221

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### Fission Energy Distributions

86-Rn-221



86-Rn-221

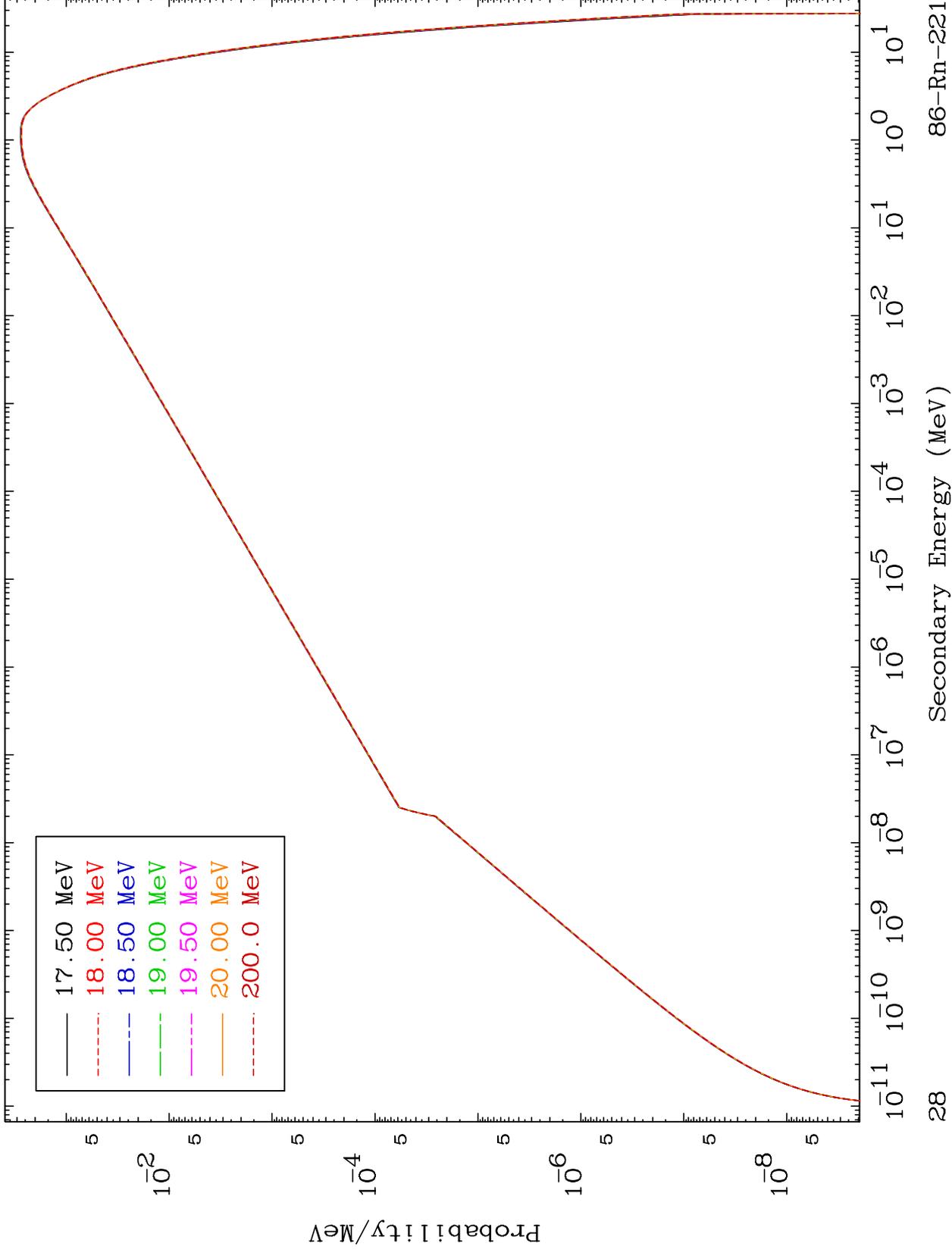
Secondary Energy (MeV)

27

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### Fission Energy Distributions

86-Rn-221



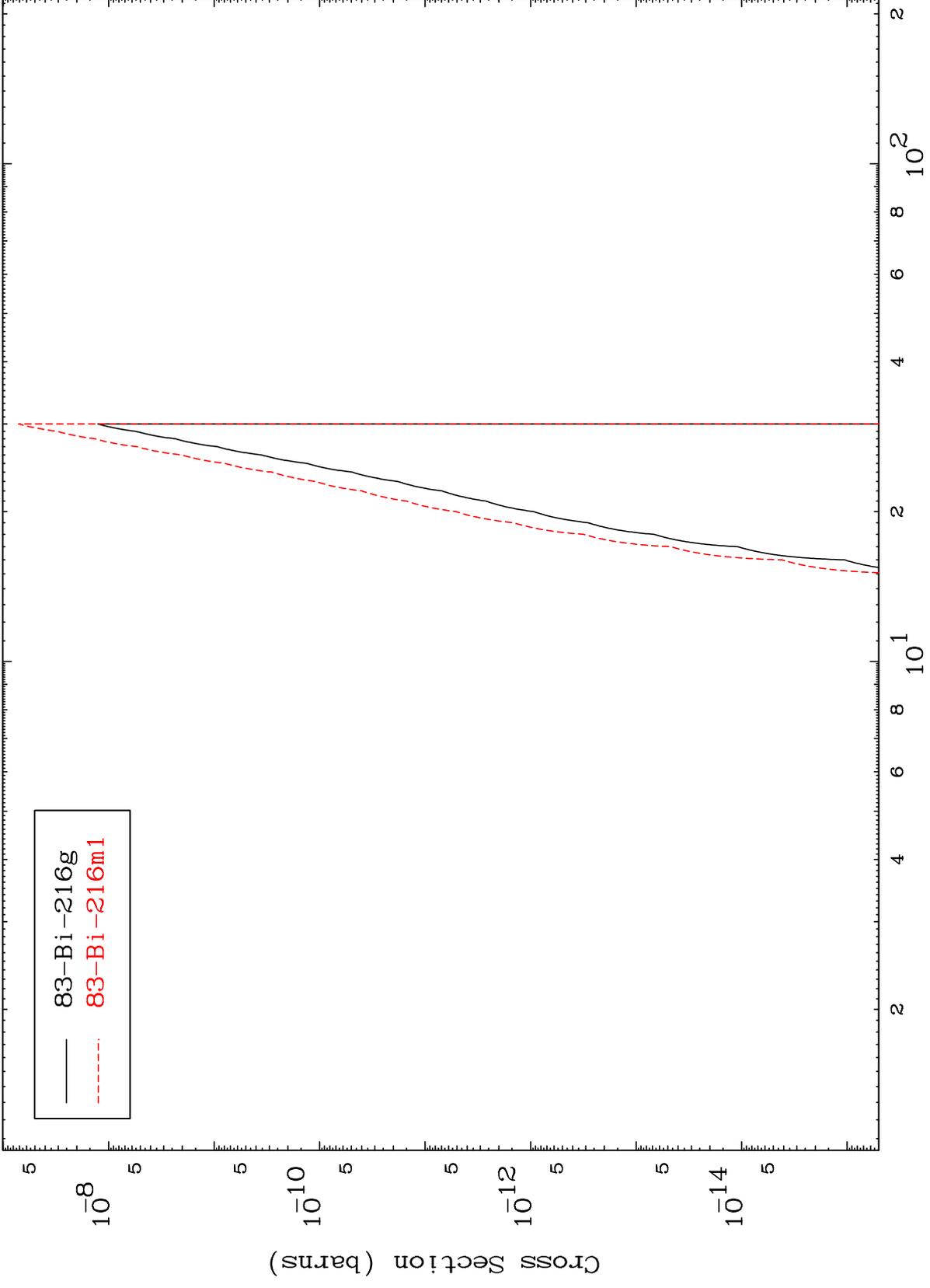
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(n,n') p  $\alpha$

86-Rn-221

Radionuclide Production Cross Section



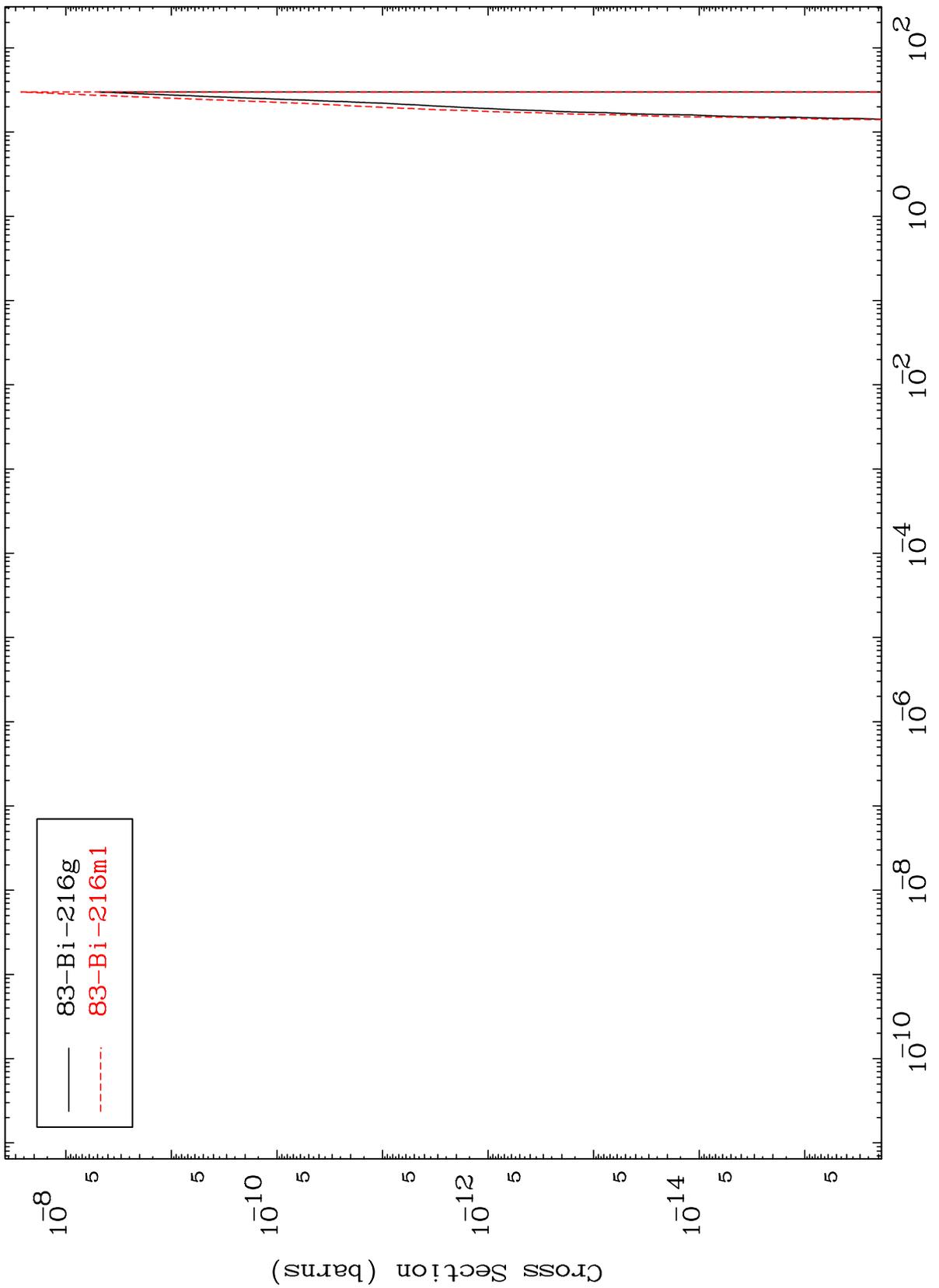
83-Bi-216g  
83-Bi-216m1

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(n,d)  $\alpha$

86-Rn-221

Radionuclide Production Cross Section



30

Incident Energy (MeV)

86-Rn-221