

Program EVALPLOT
(Version 2021-1)

by

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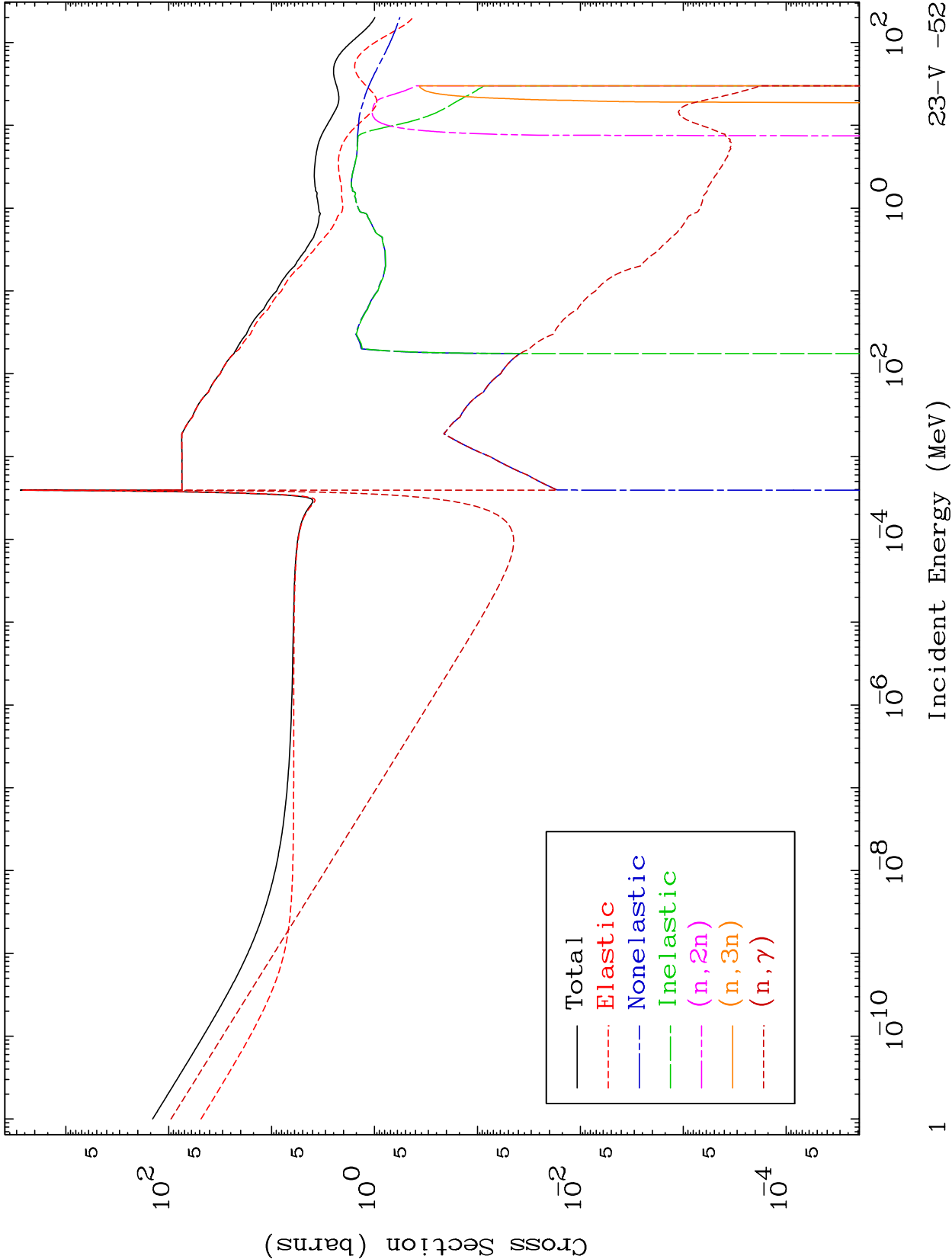
E.Mail: redcullen1@comcast.net
Web: redcullen1.net/HOMEPAGE.NEW

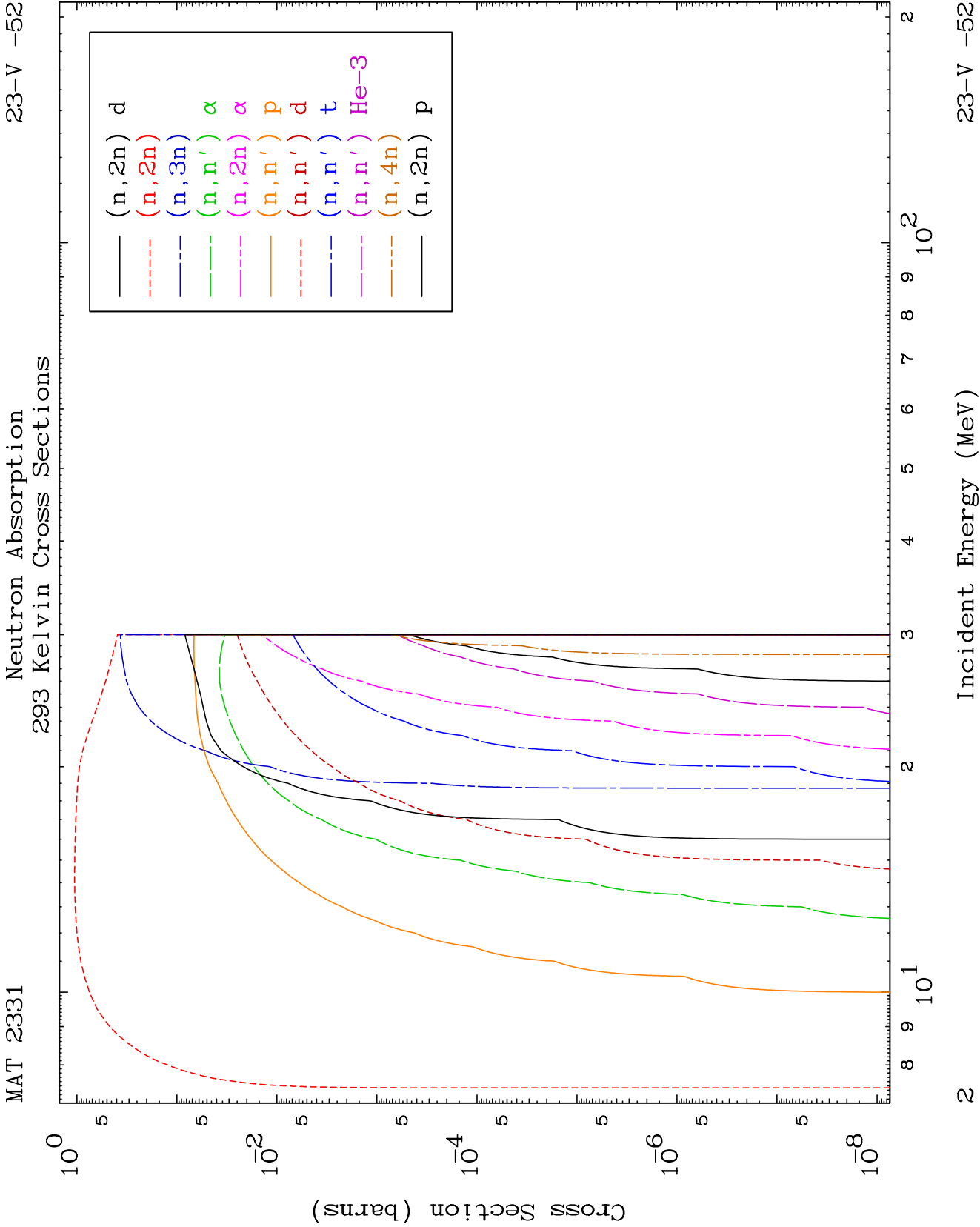
Press Mouse Button to Start

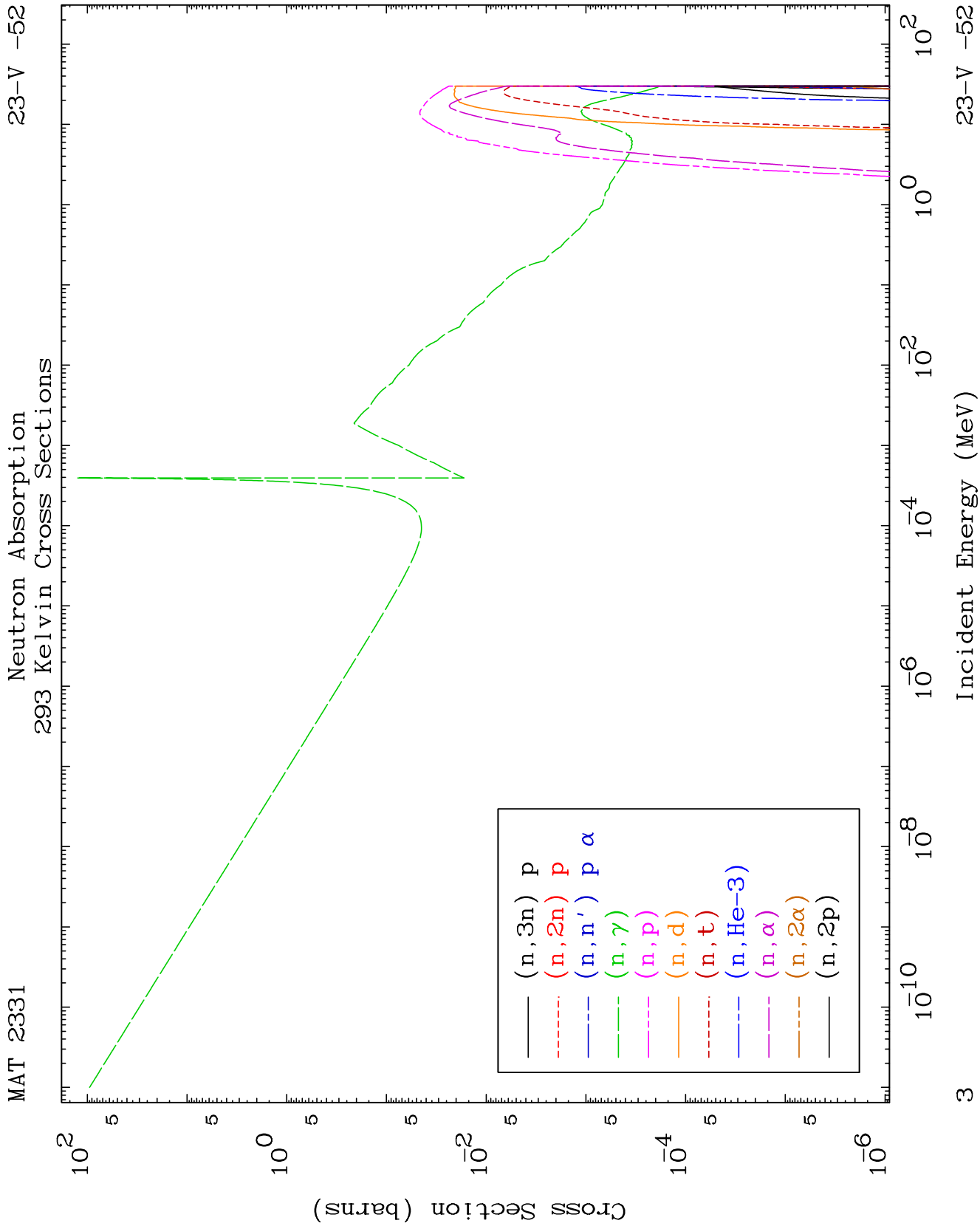
MAT 2331

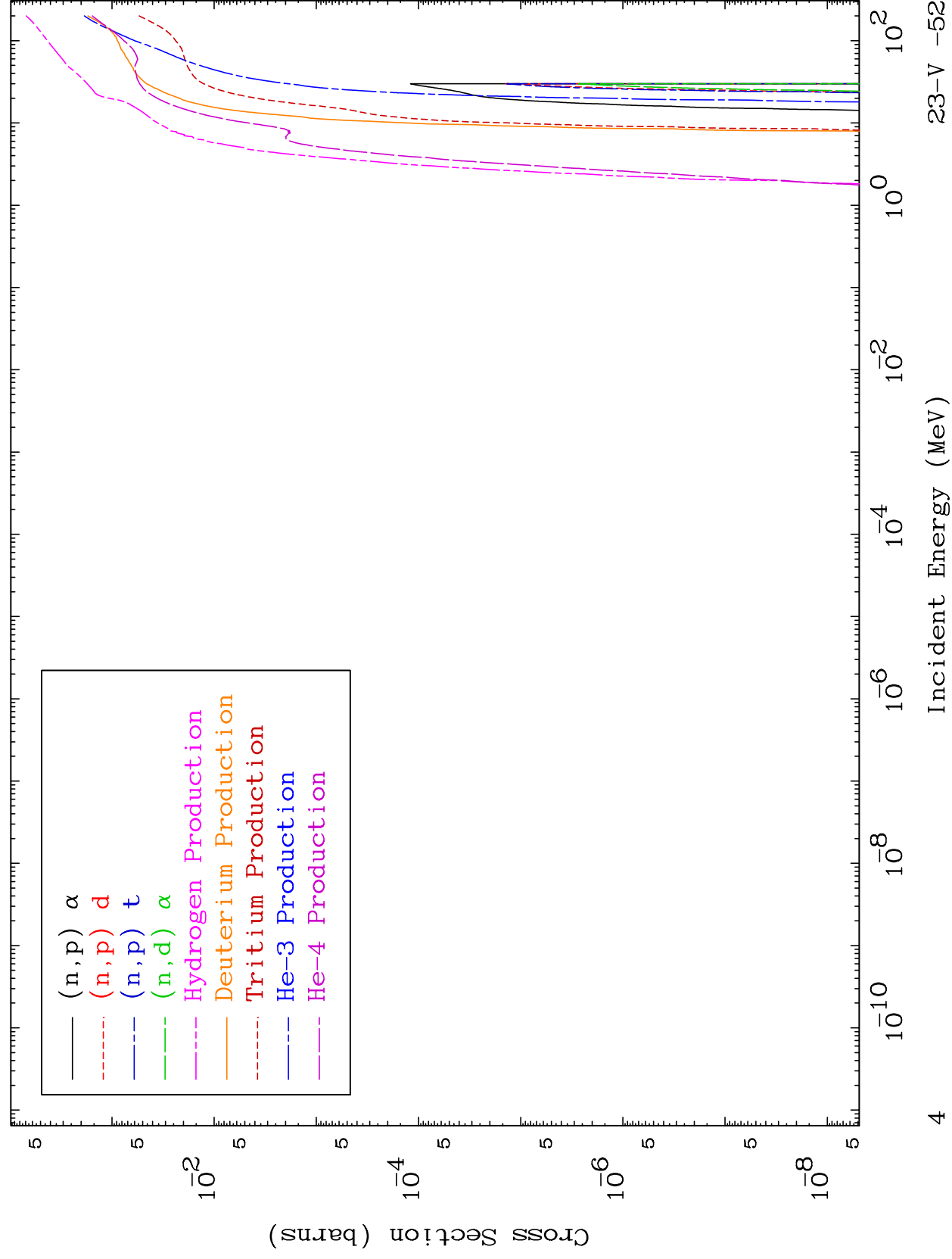
Neutron Major
293 Kelvin Cross Sections

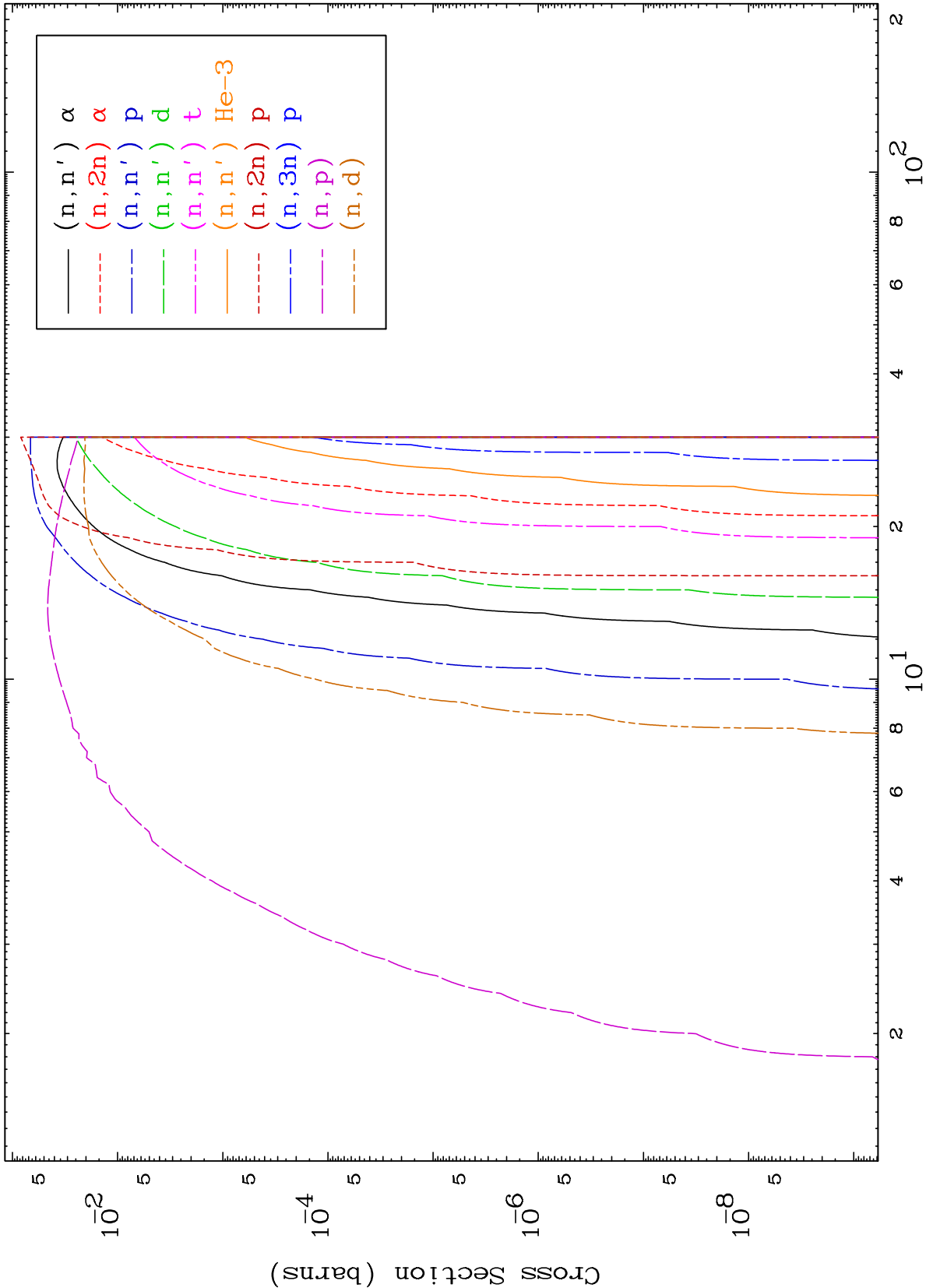
23-V -52

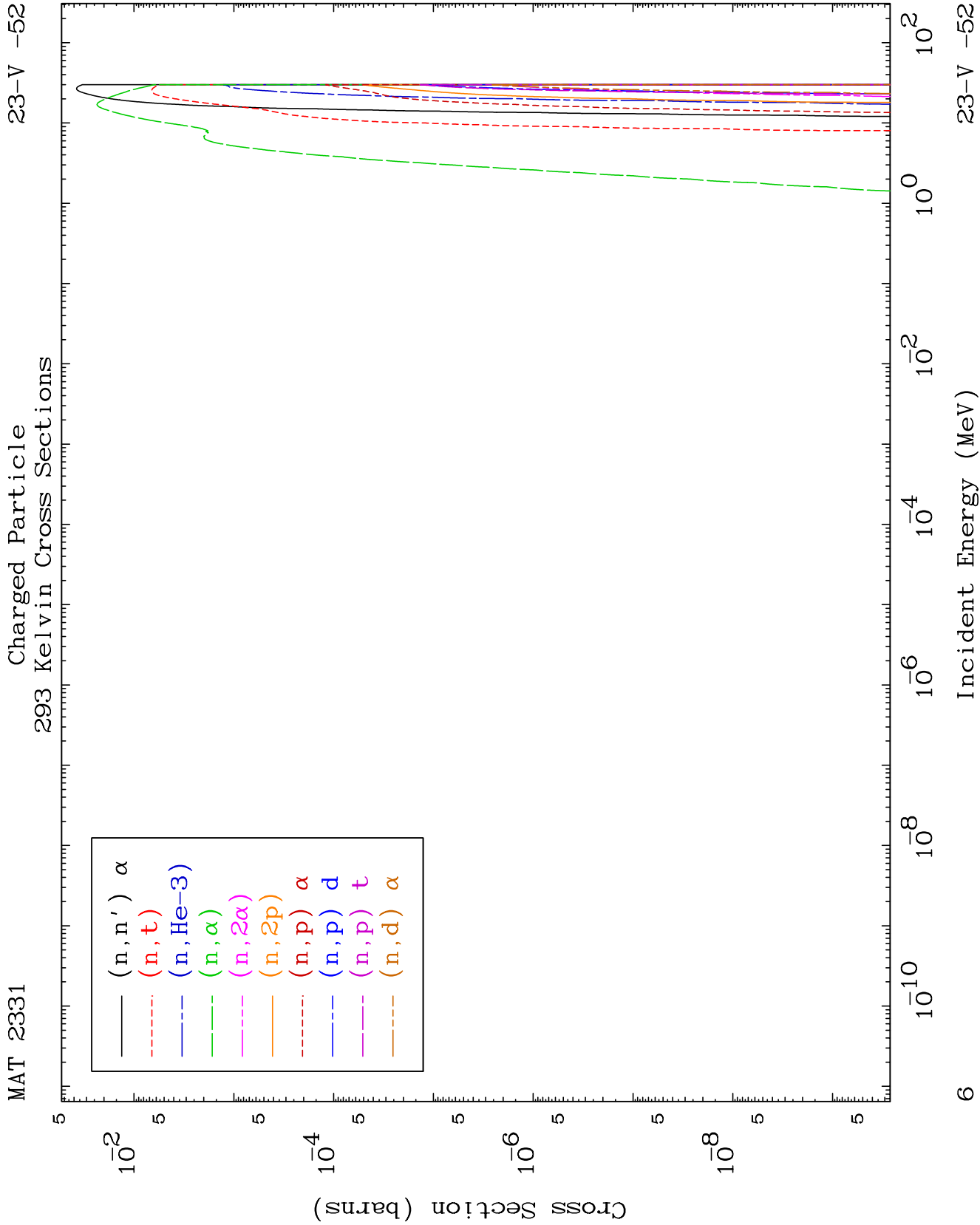








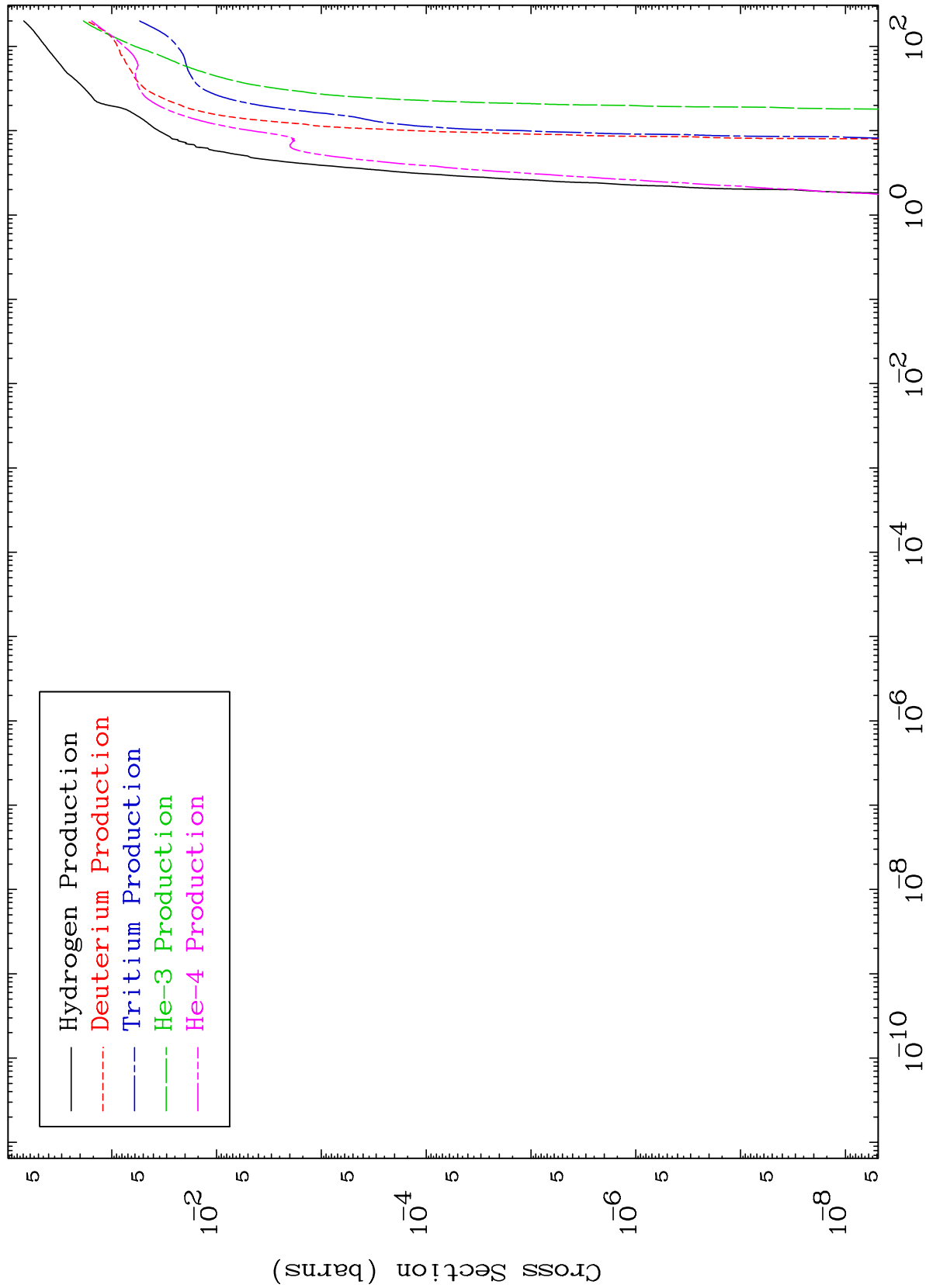




MAT 2331

Particle Production
293 Kelvin Cross Sections

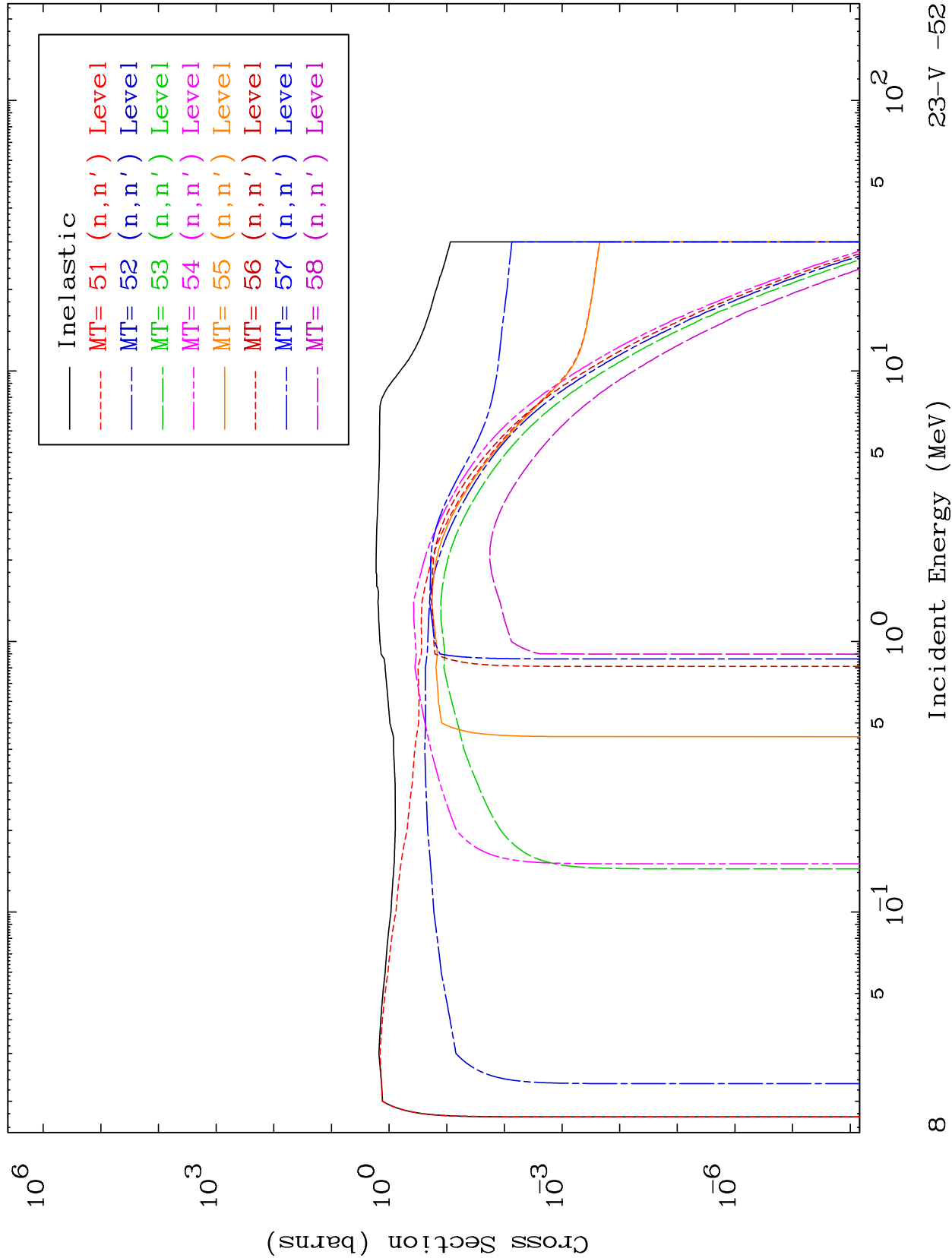
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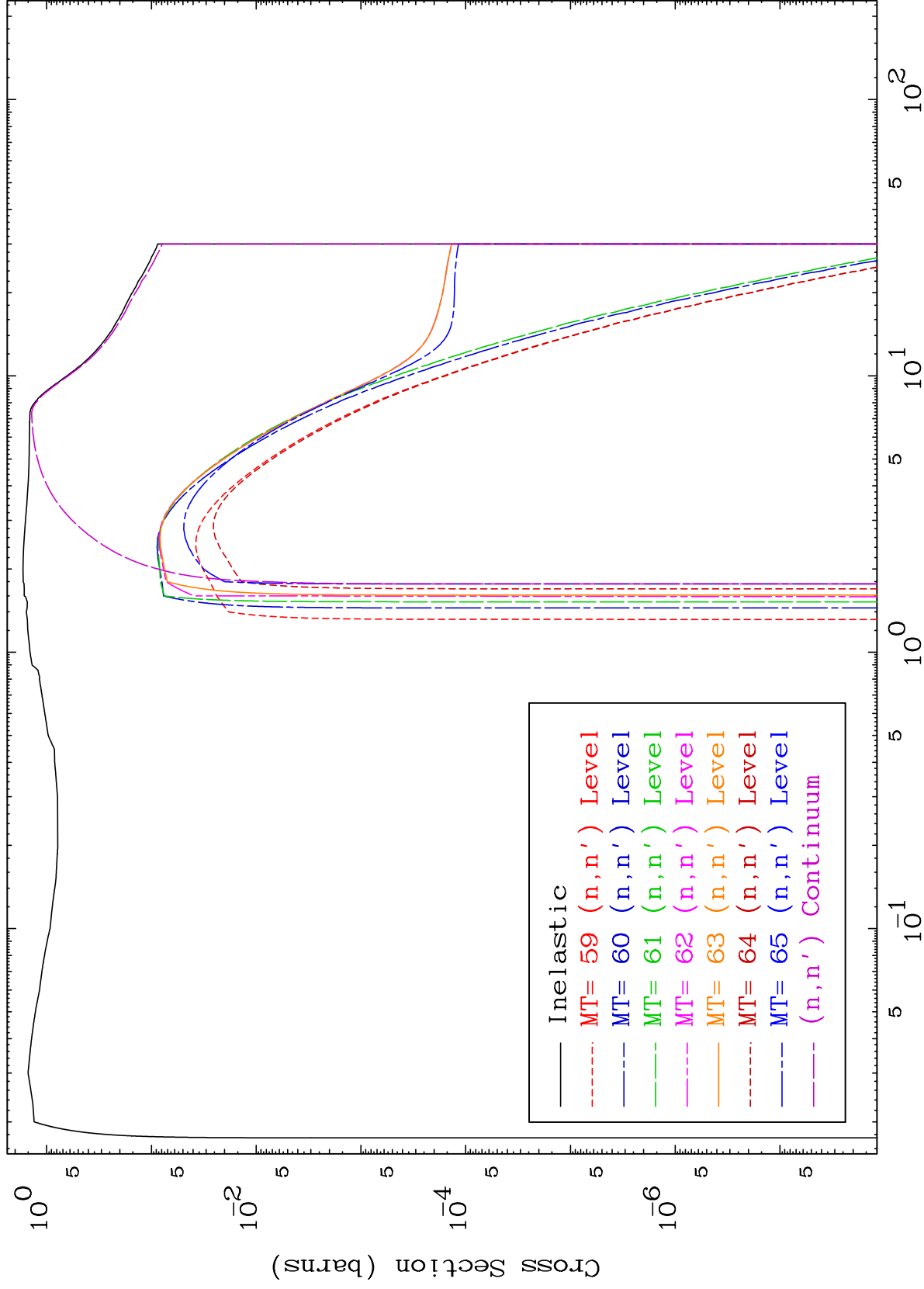


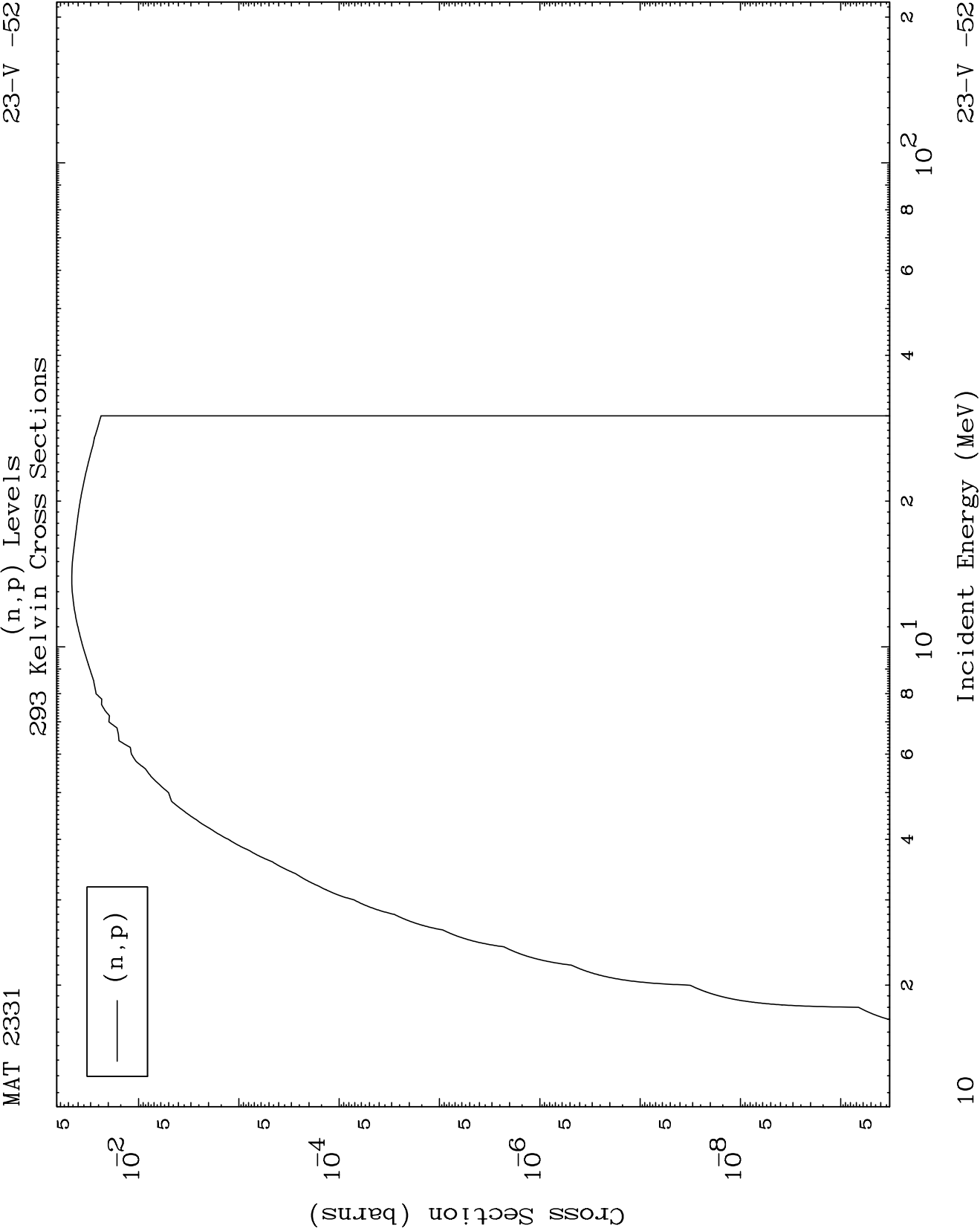
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Incident Energy (MeV)

23-V -52



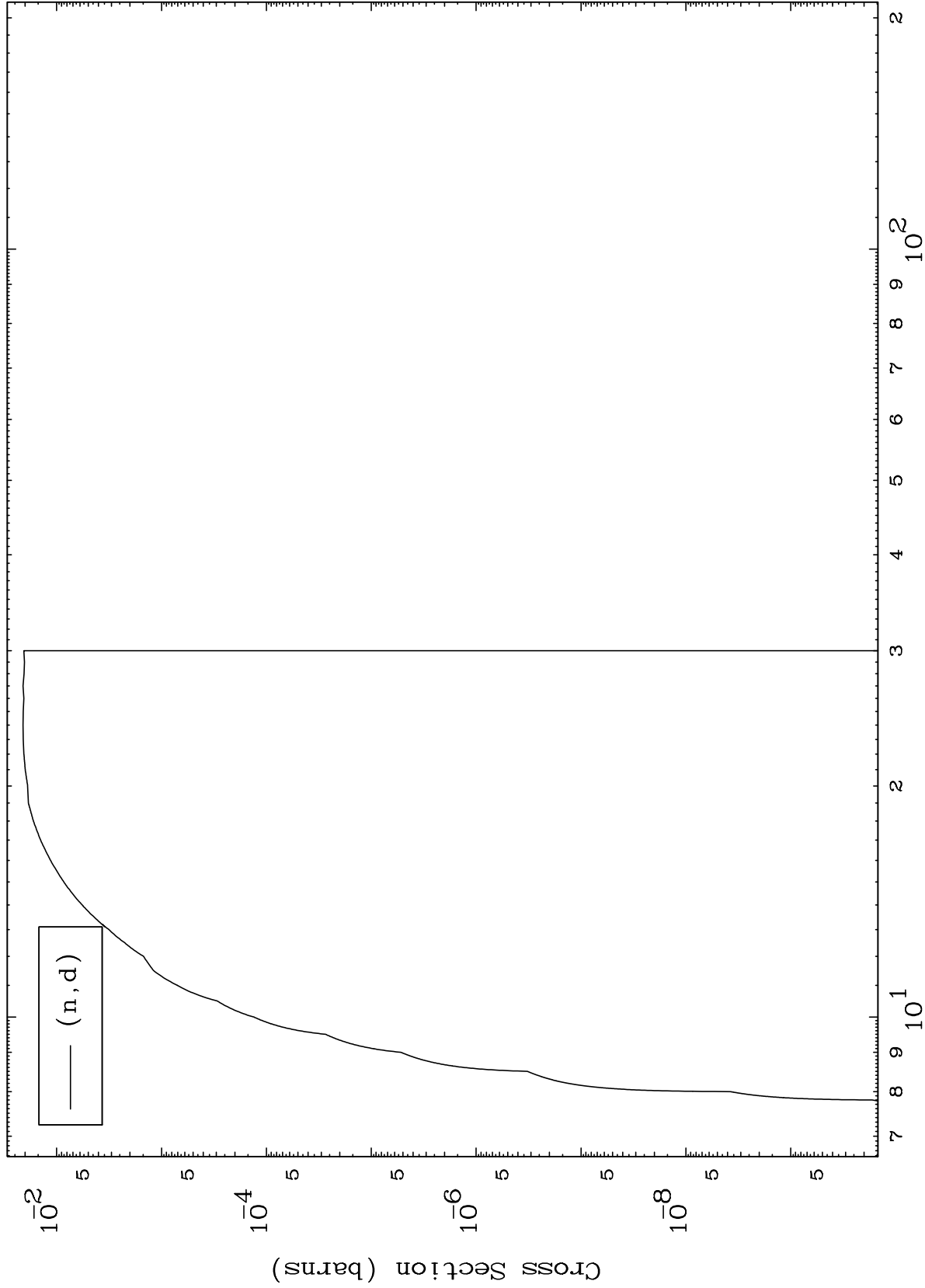
(n,n') Levels
293 Kelvin Cross Sections



MAT 2331

(n,d) Levels
293 Kelvin Cross Sections

23-V -52



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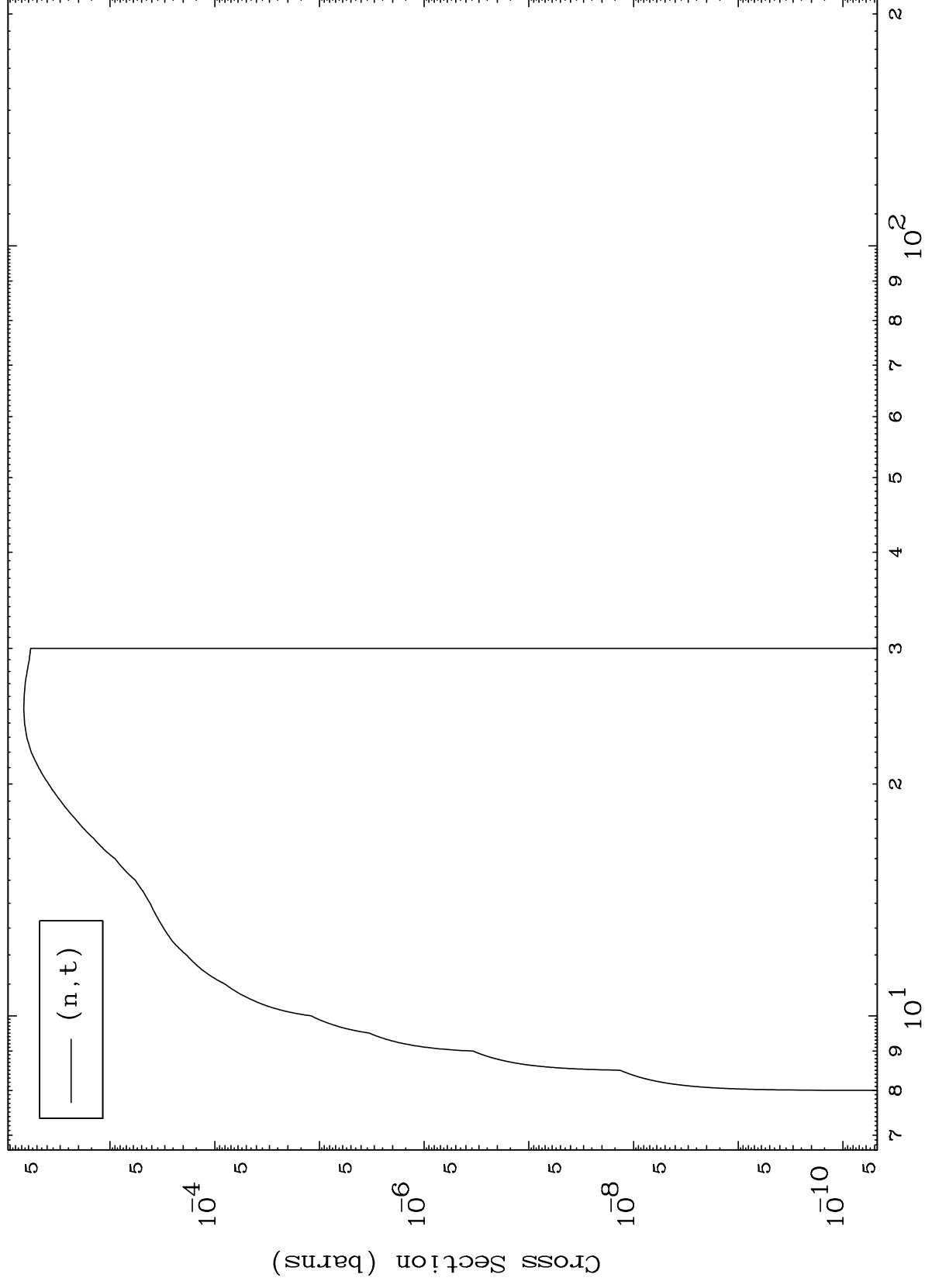
Incident Energy (MeV)

23-V -52

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(n,t) Levels
293 Kelvin Cross Sections

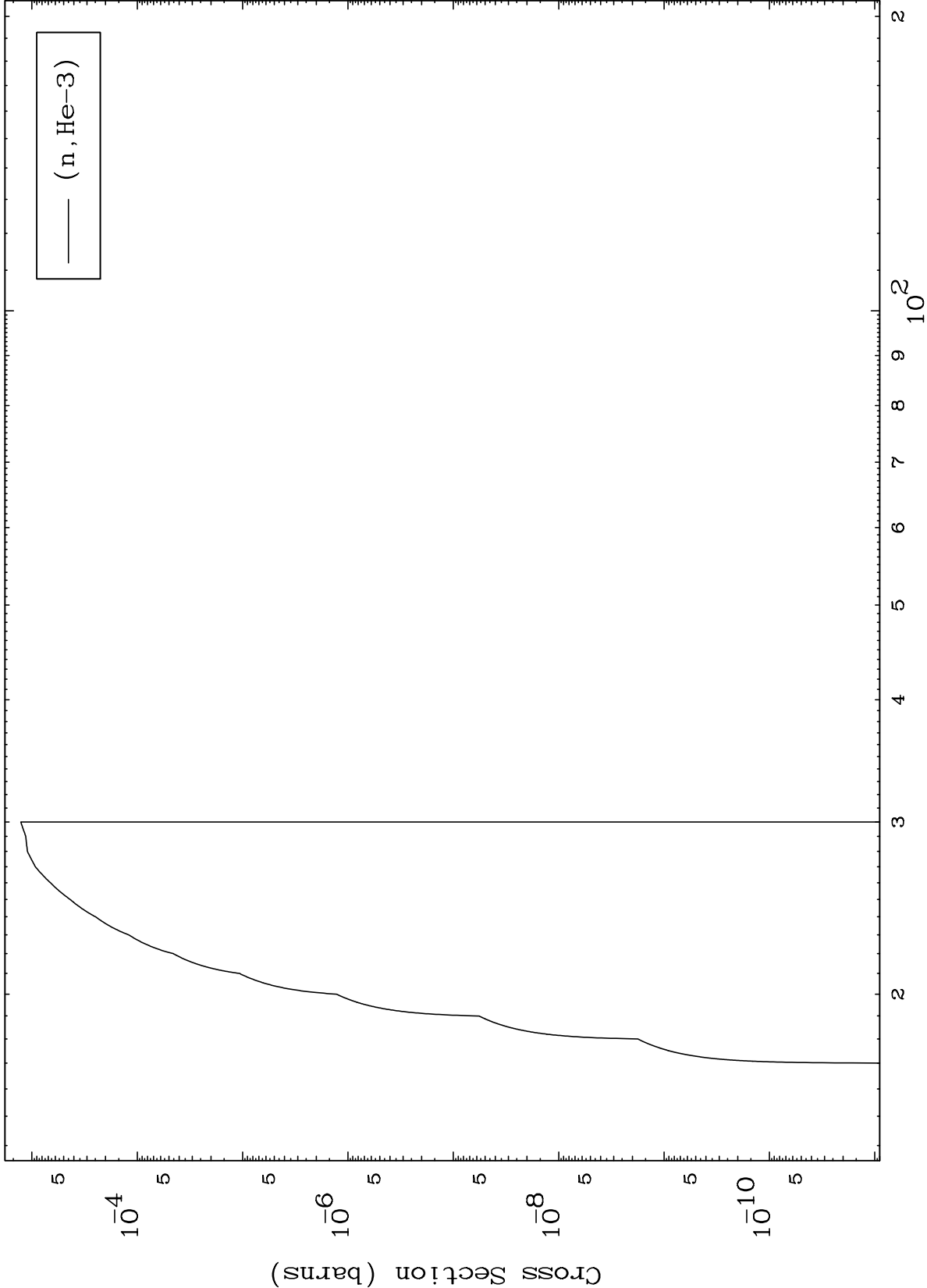
23-V -52



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Incident Energy (MeV)

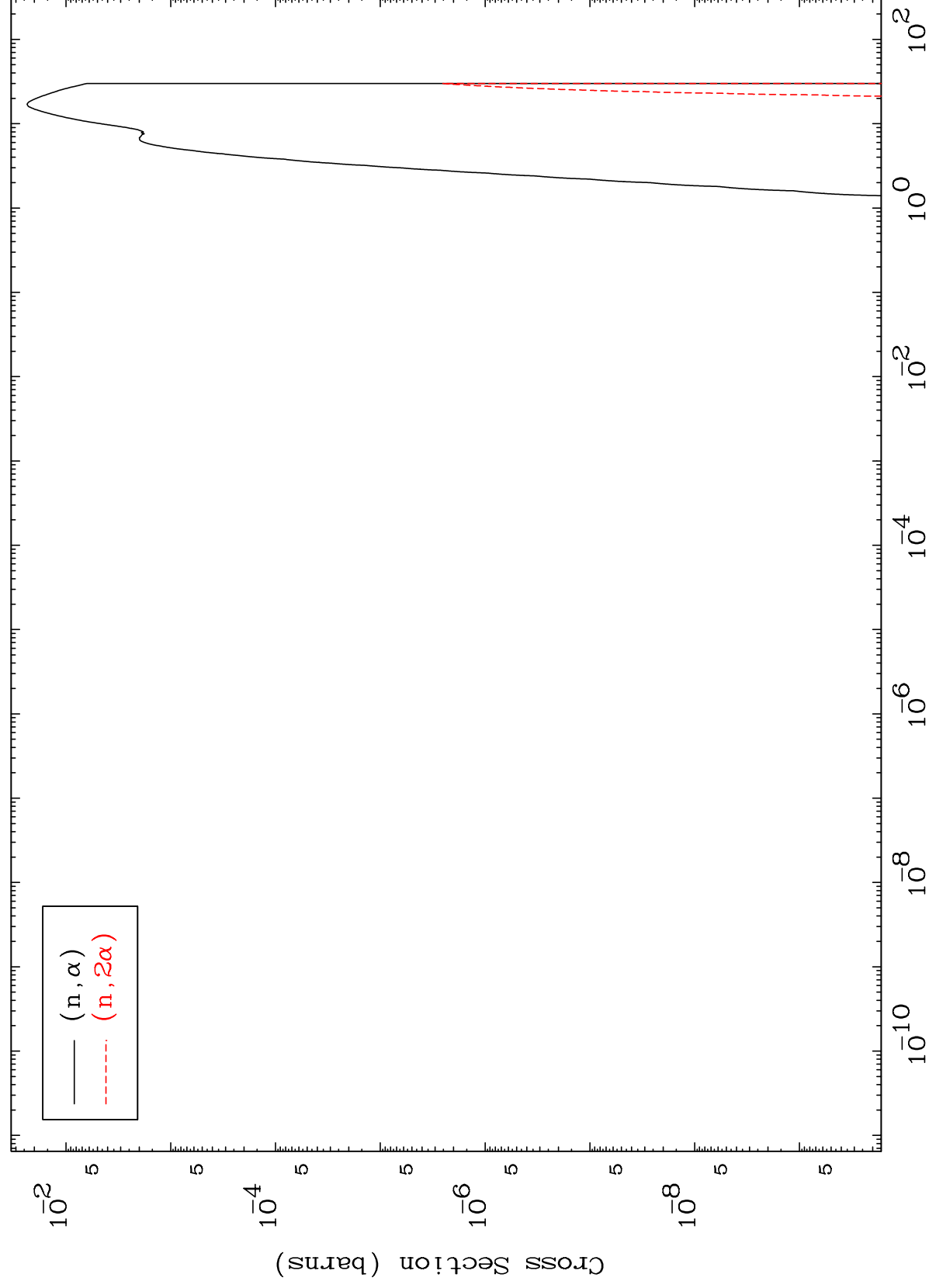
23-V -52



MAT 2331

(n, α) Levels
293 Kelvin Cross Sections

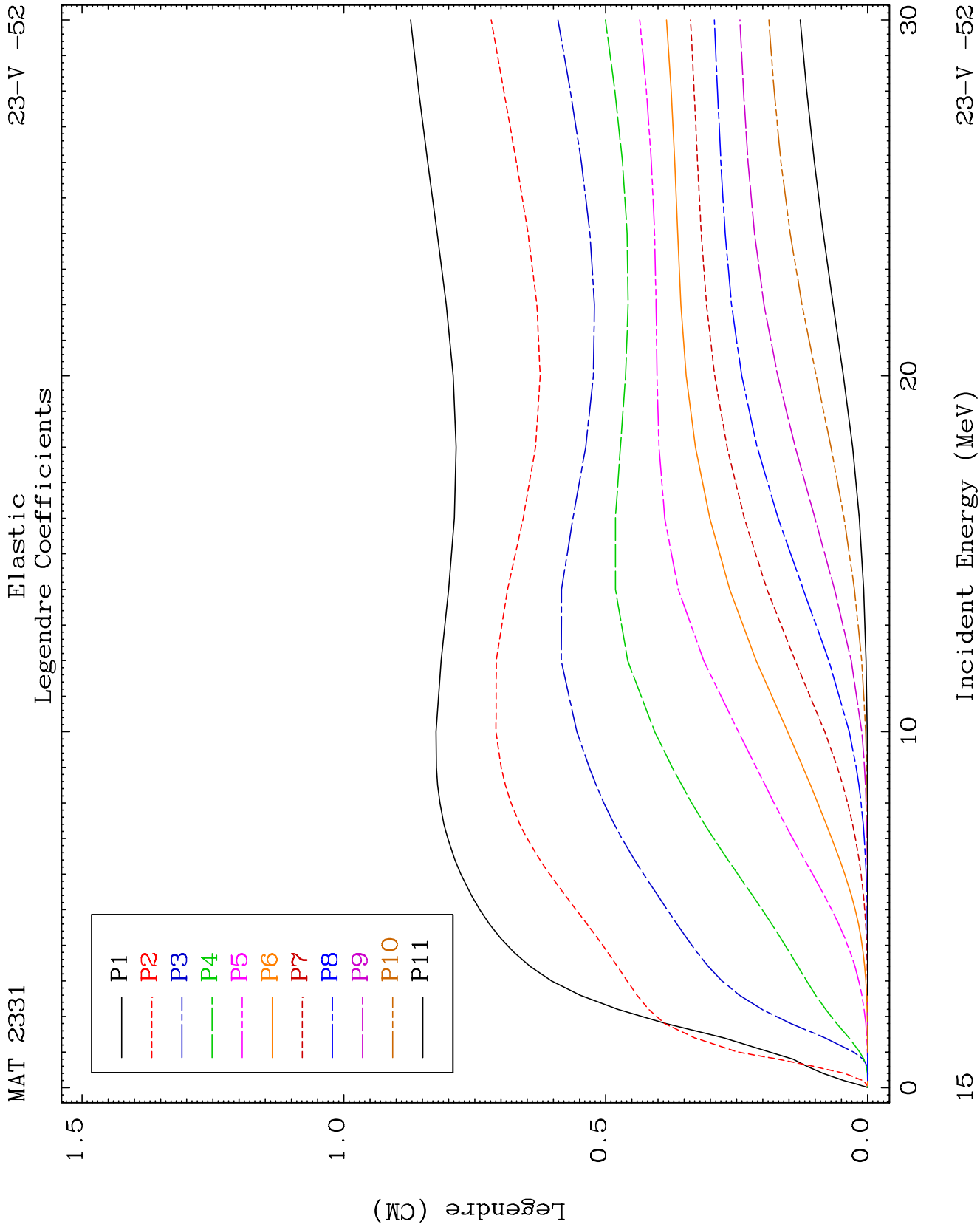
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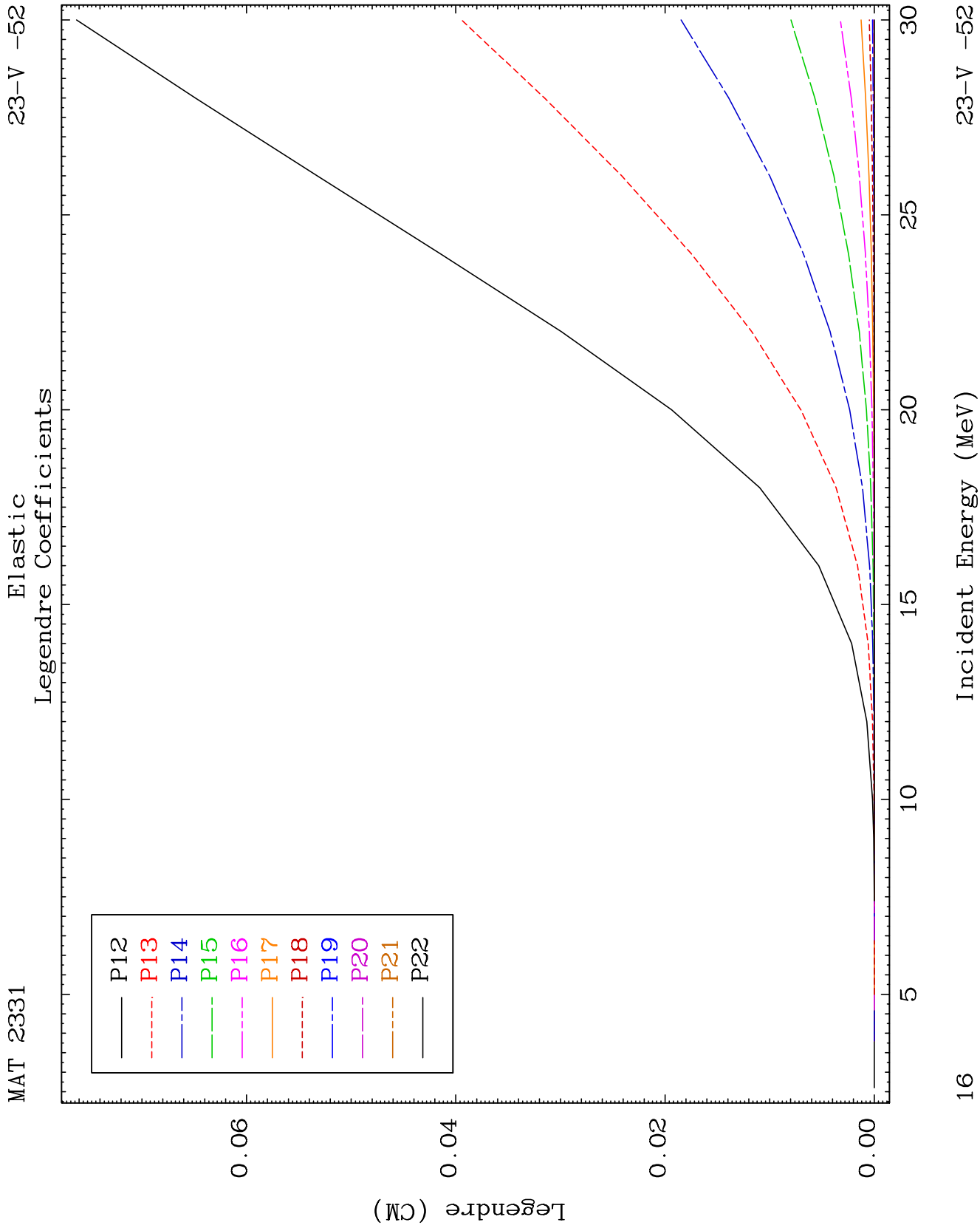


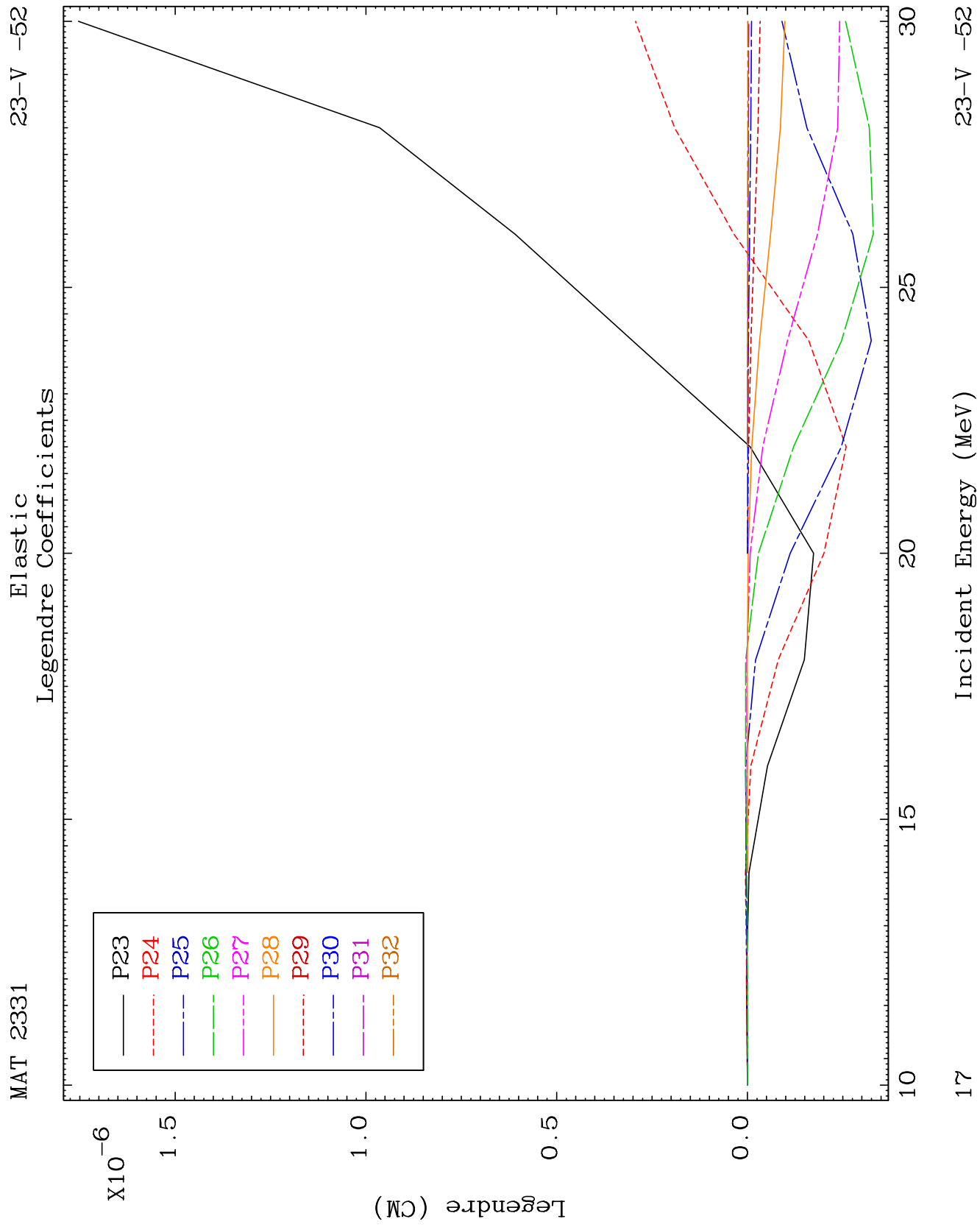
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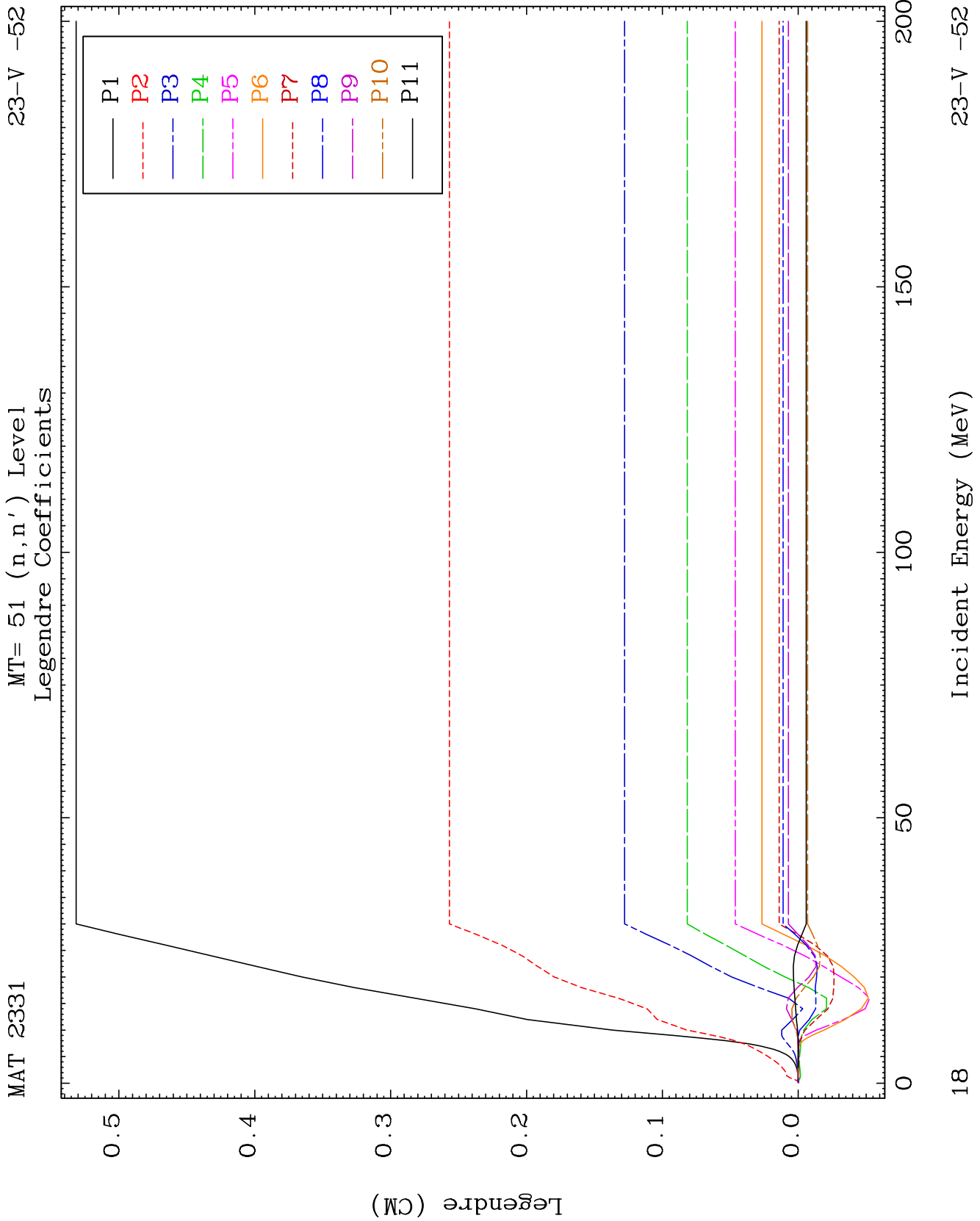
Incident Energy (MeV)

23-V -52





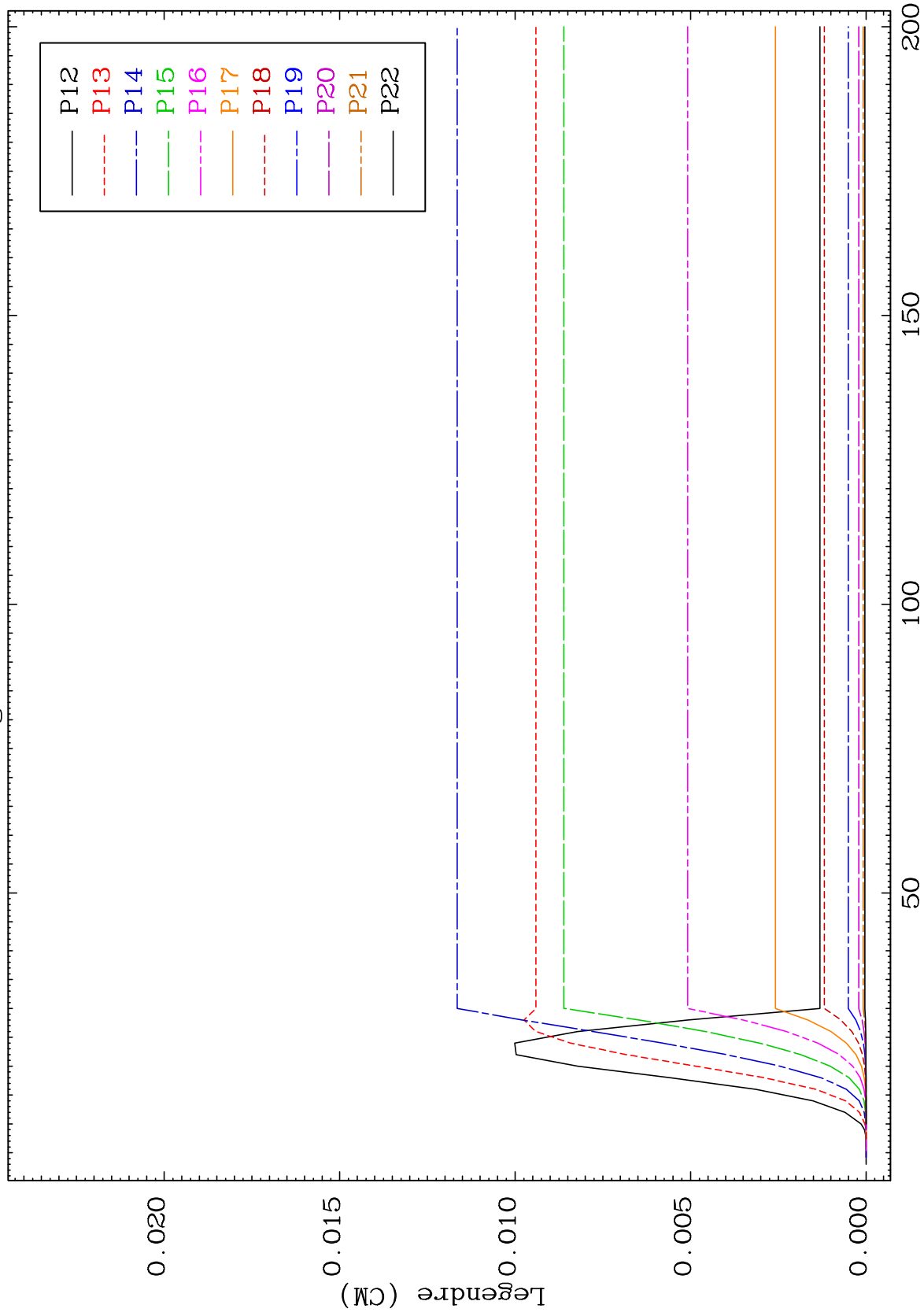




MAT 2331

MT= 51 (n,n') Level
Legendre Coefficients

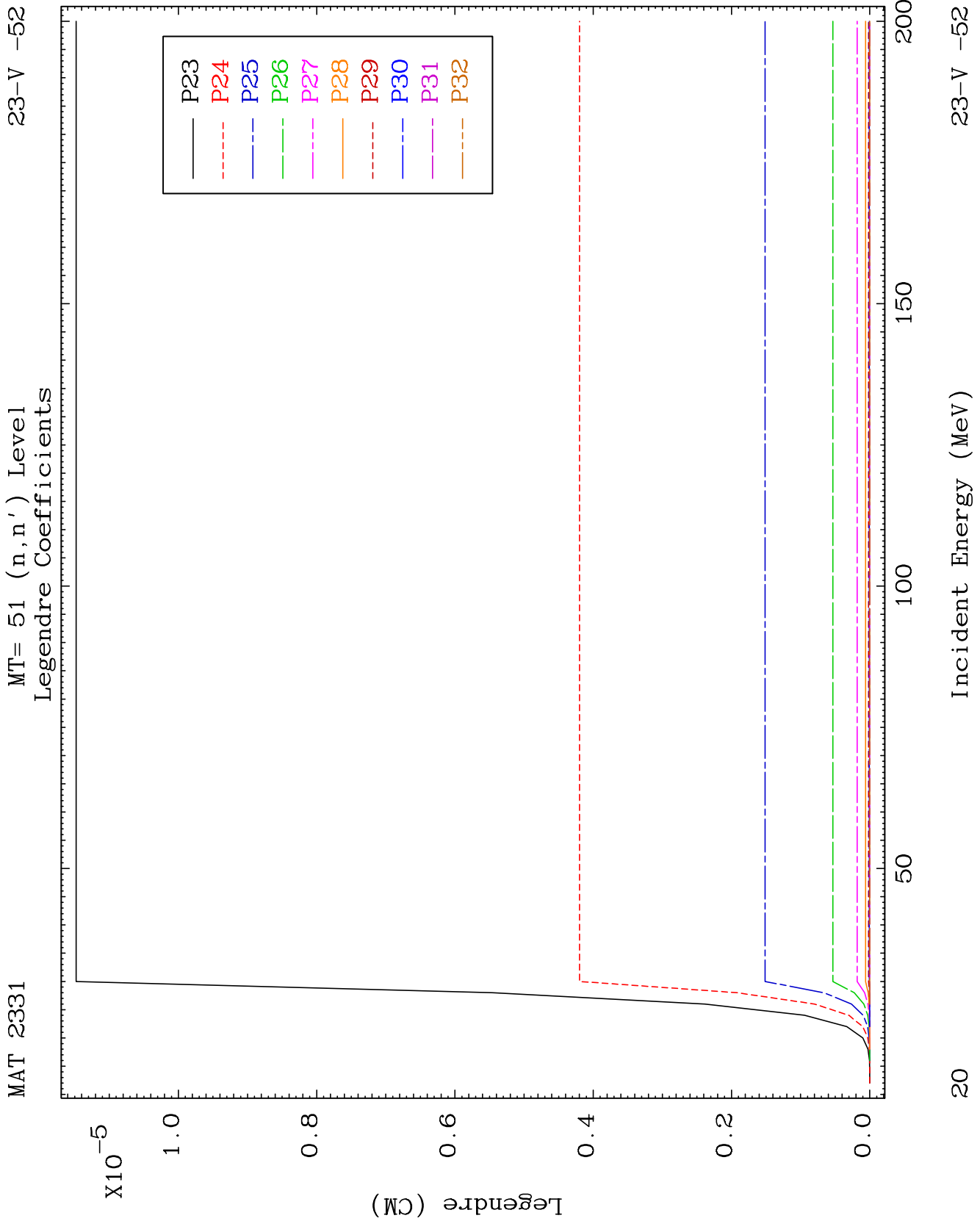
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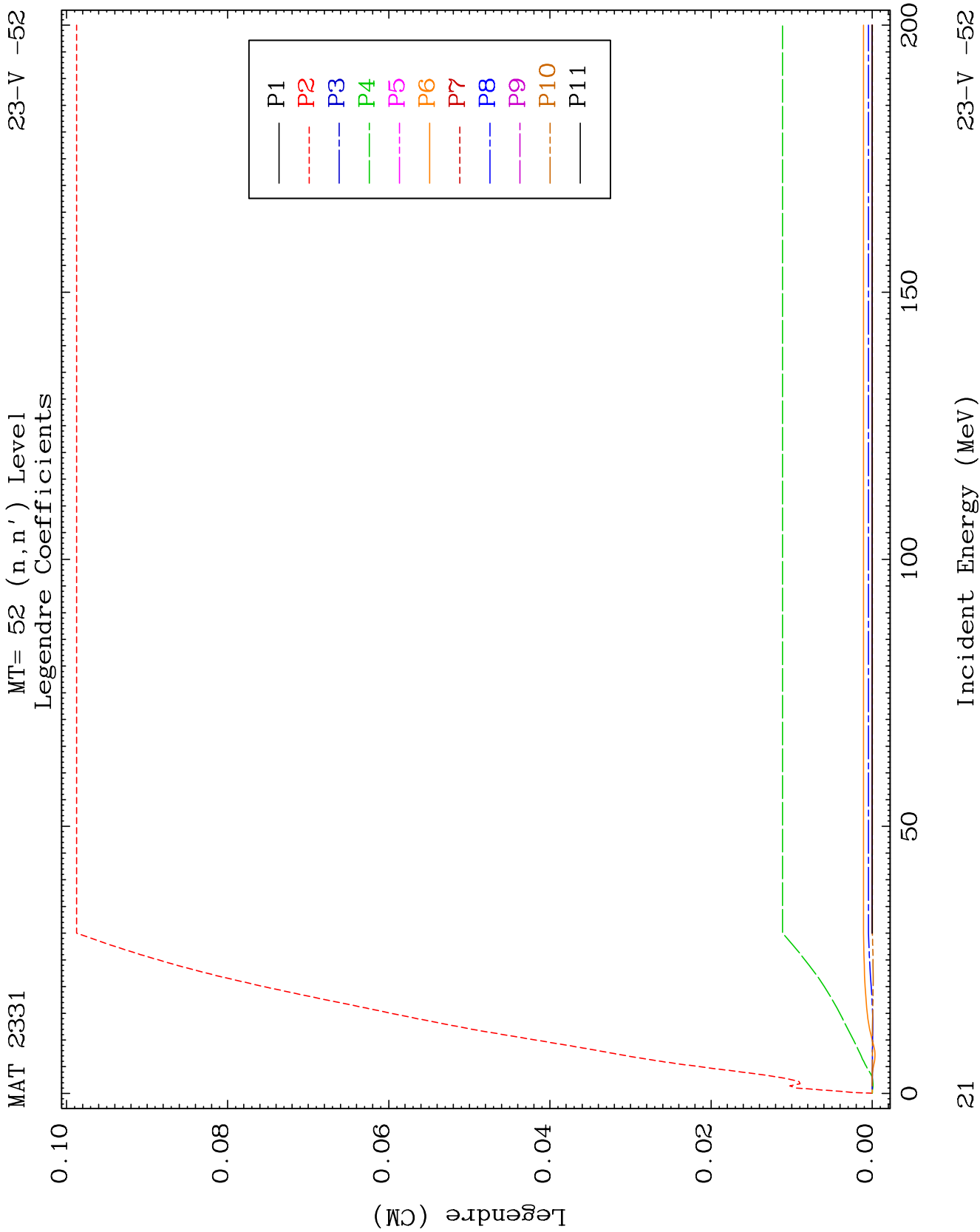


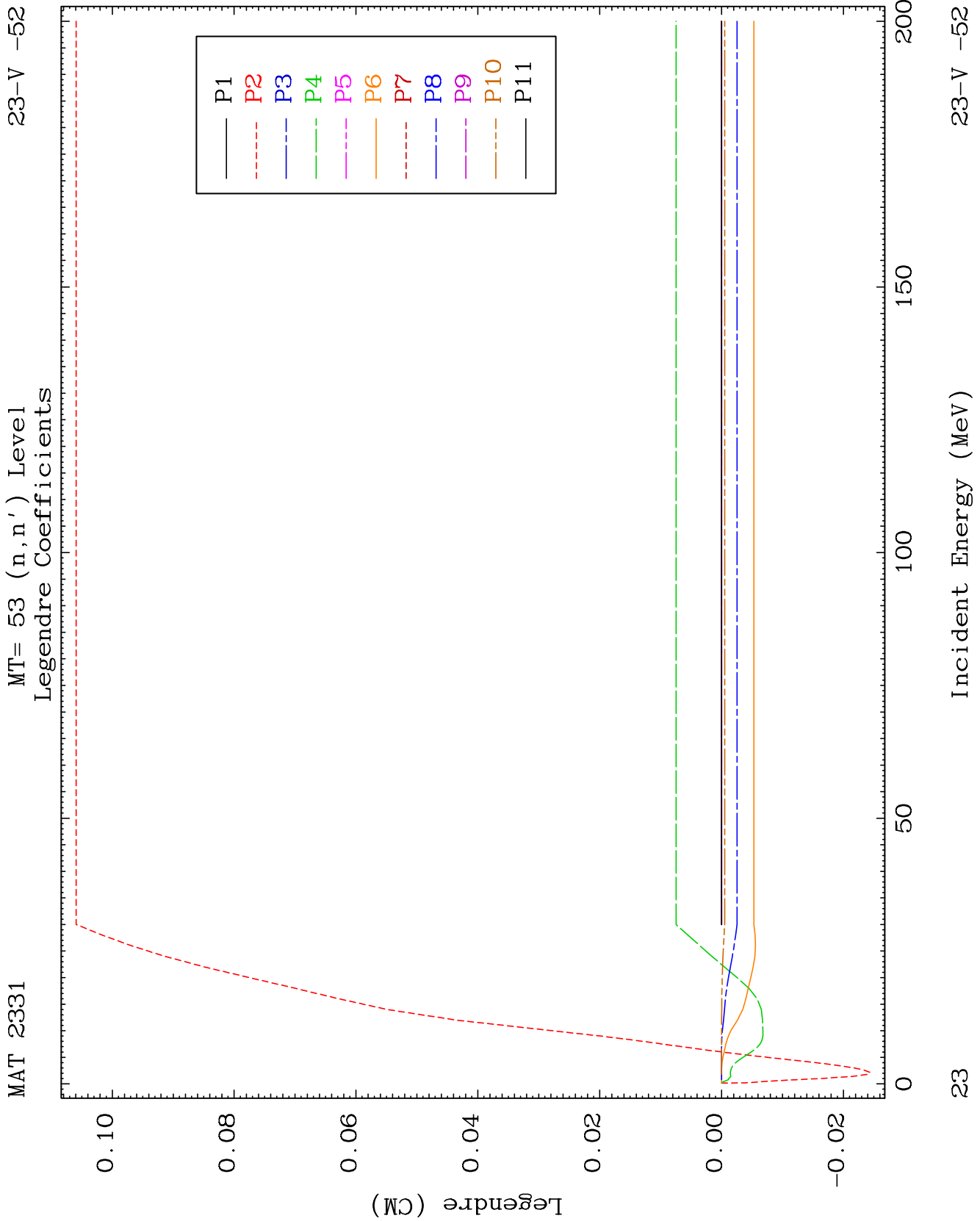
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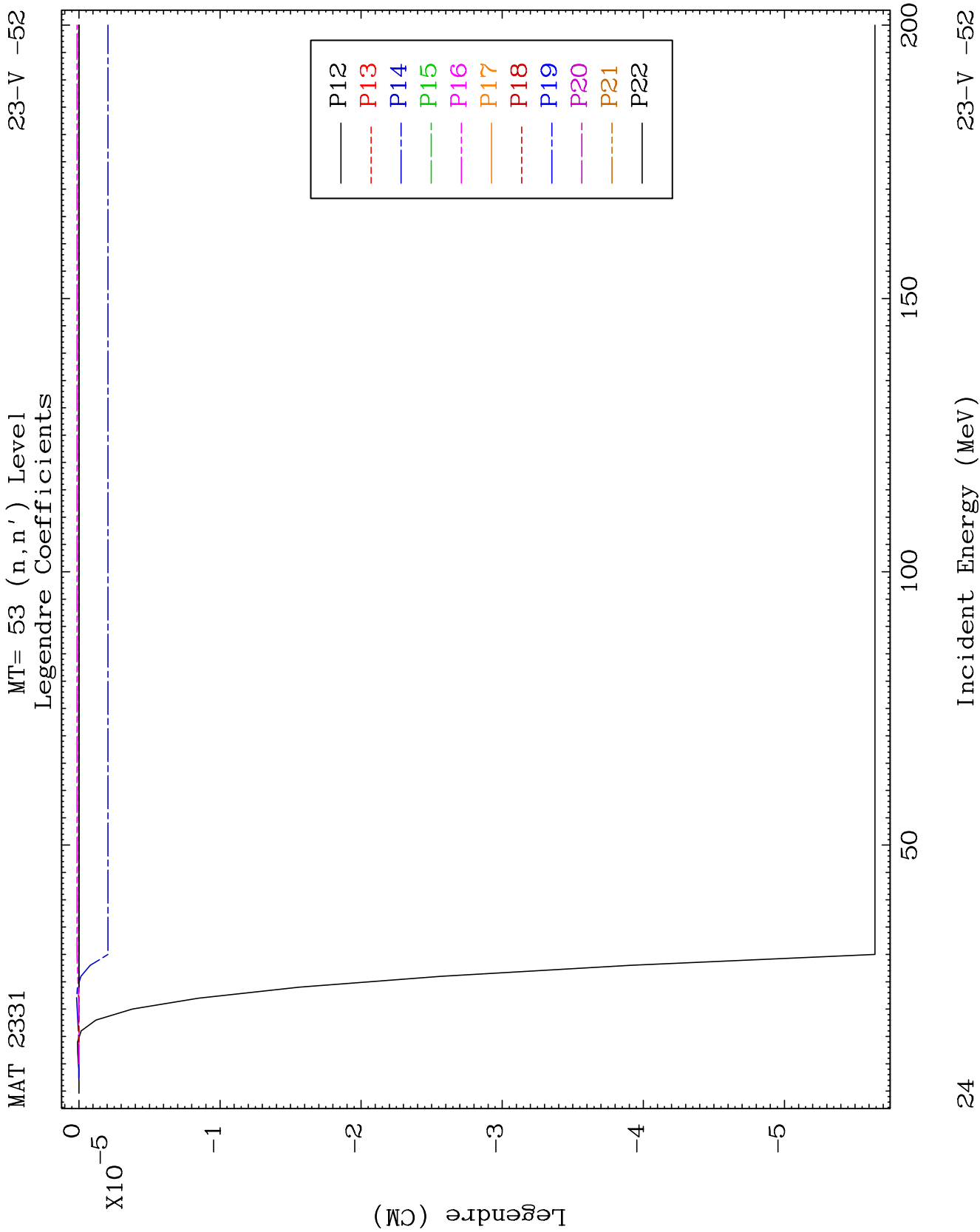
Incident Energy (MeV)

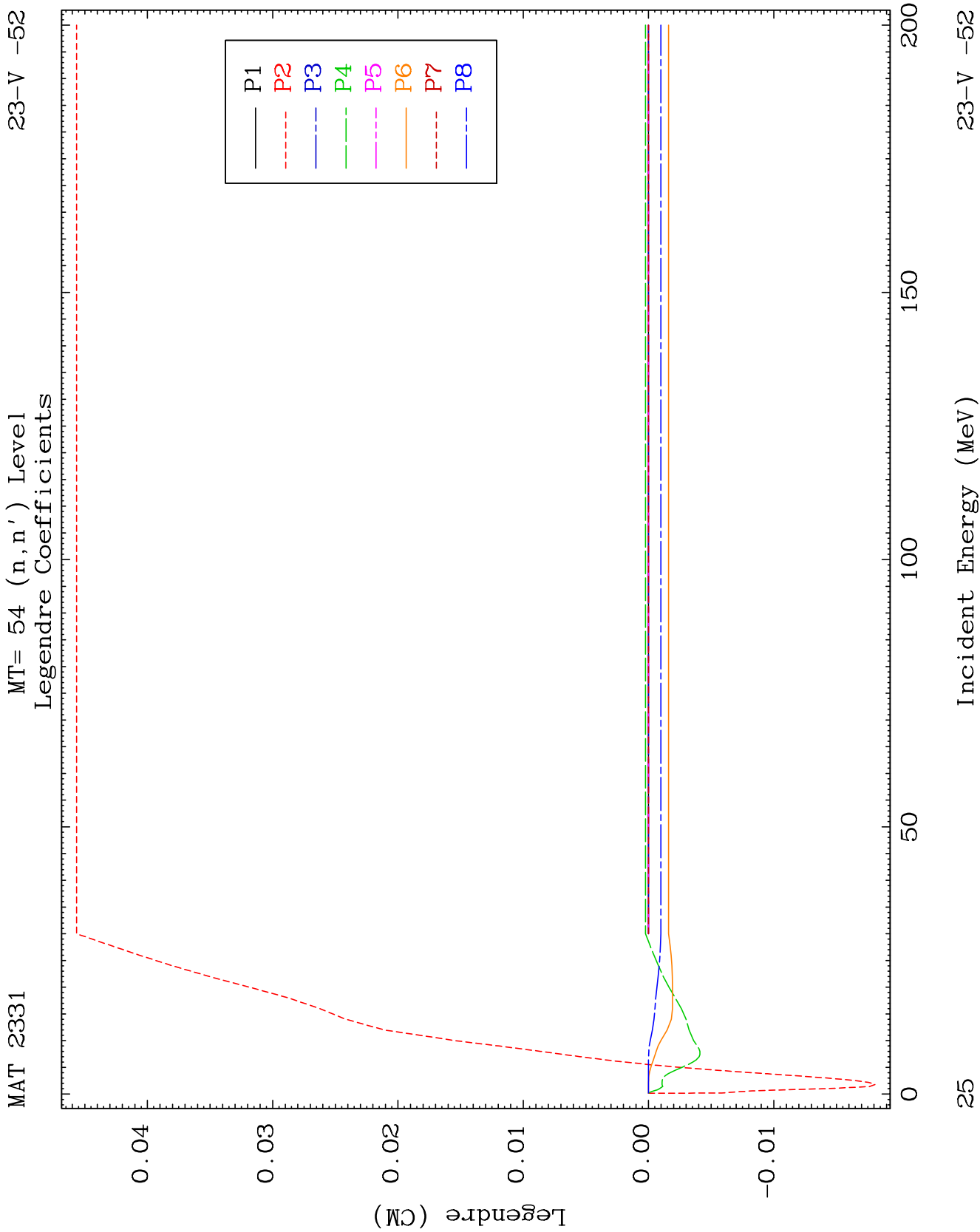
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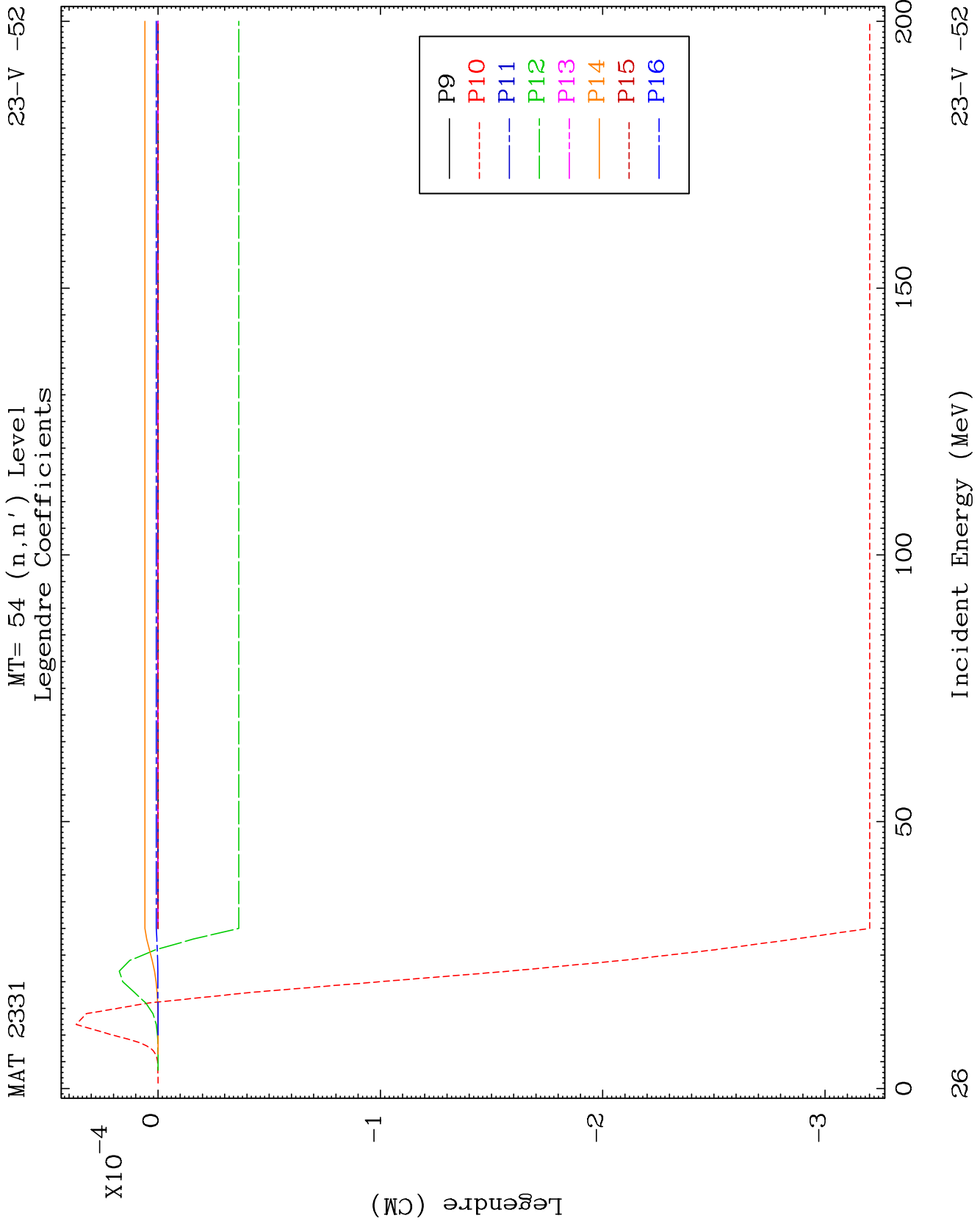


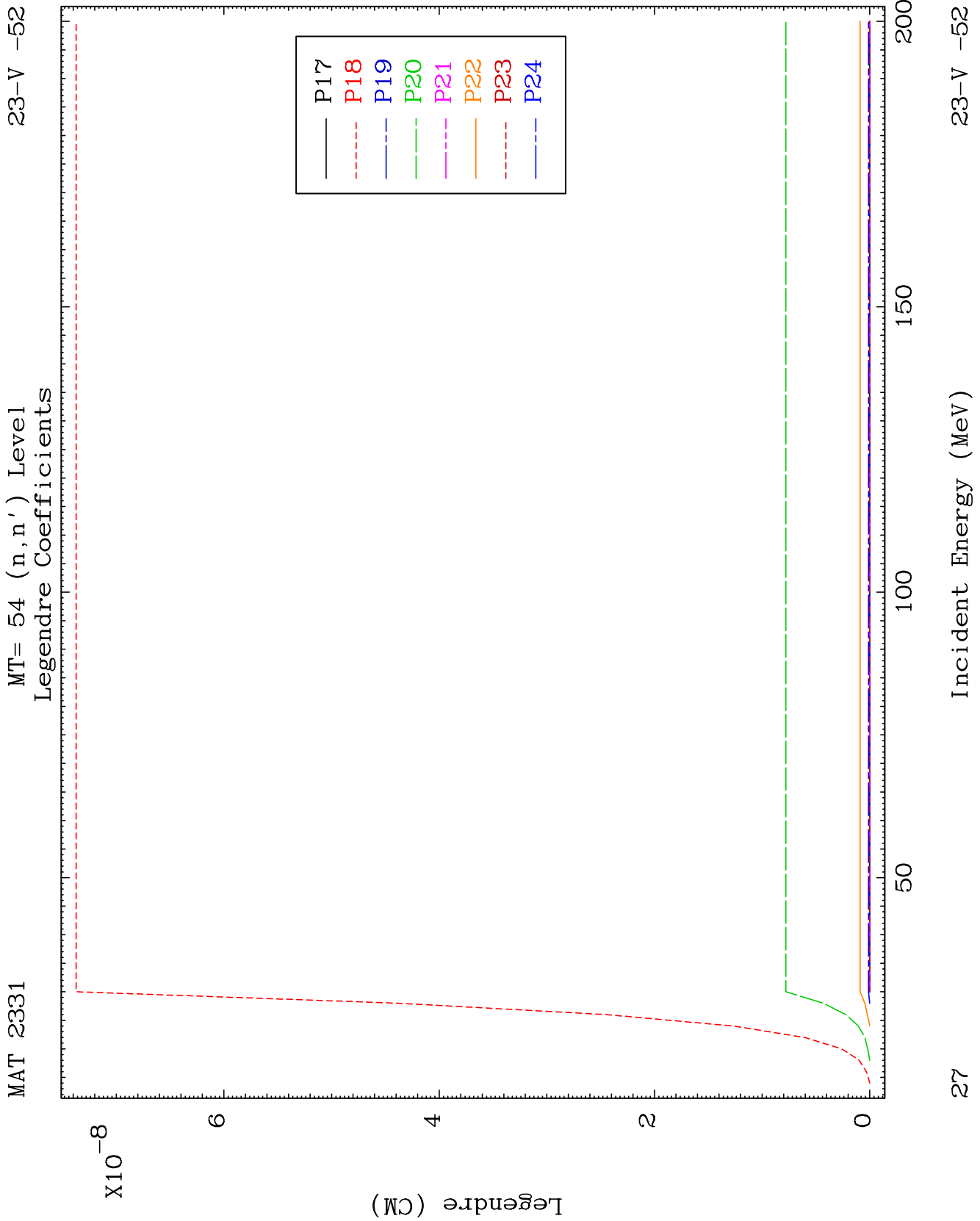


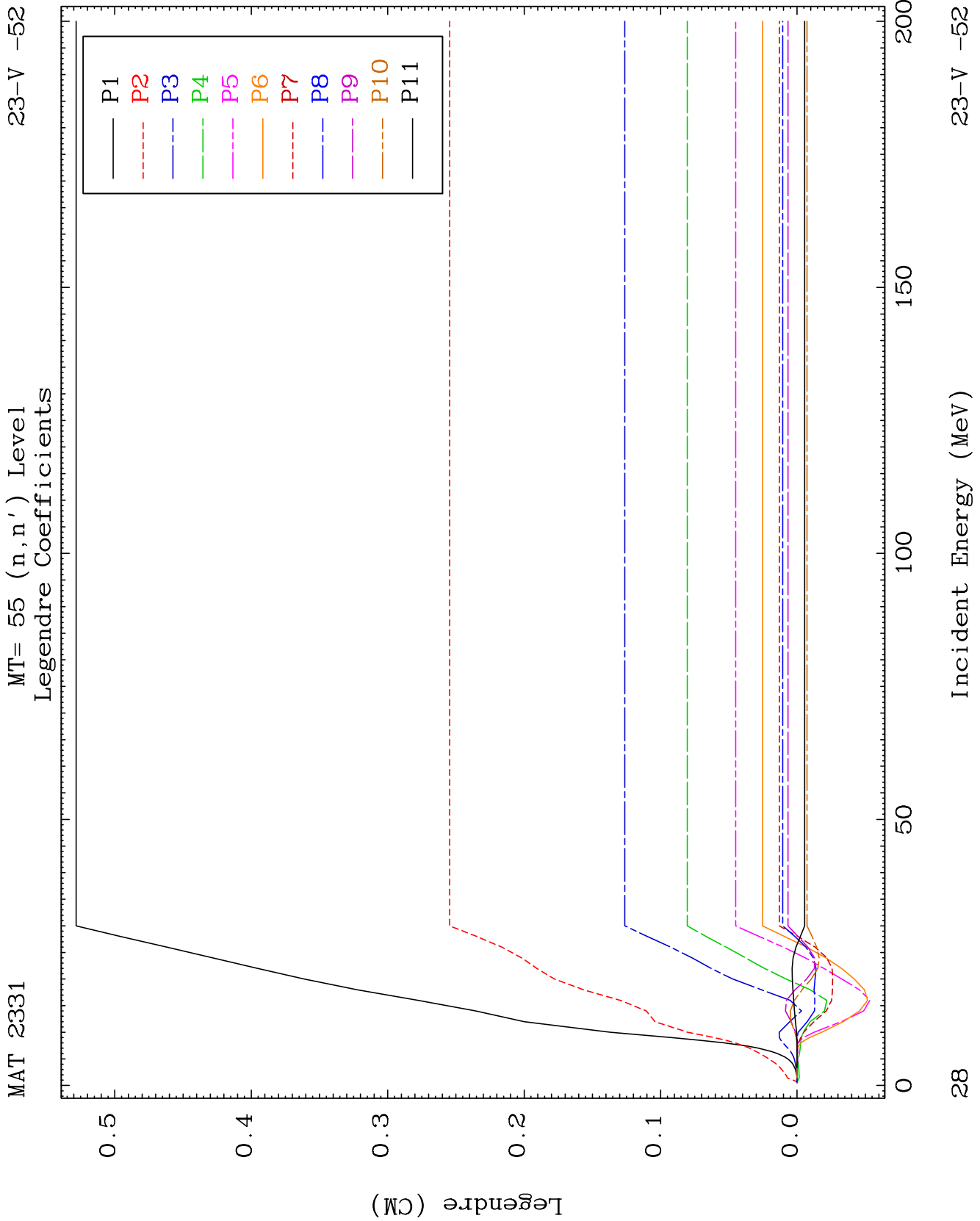


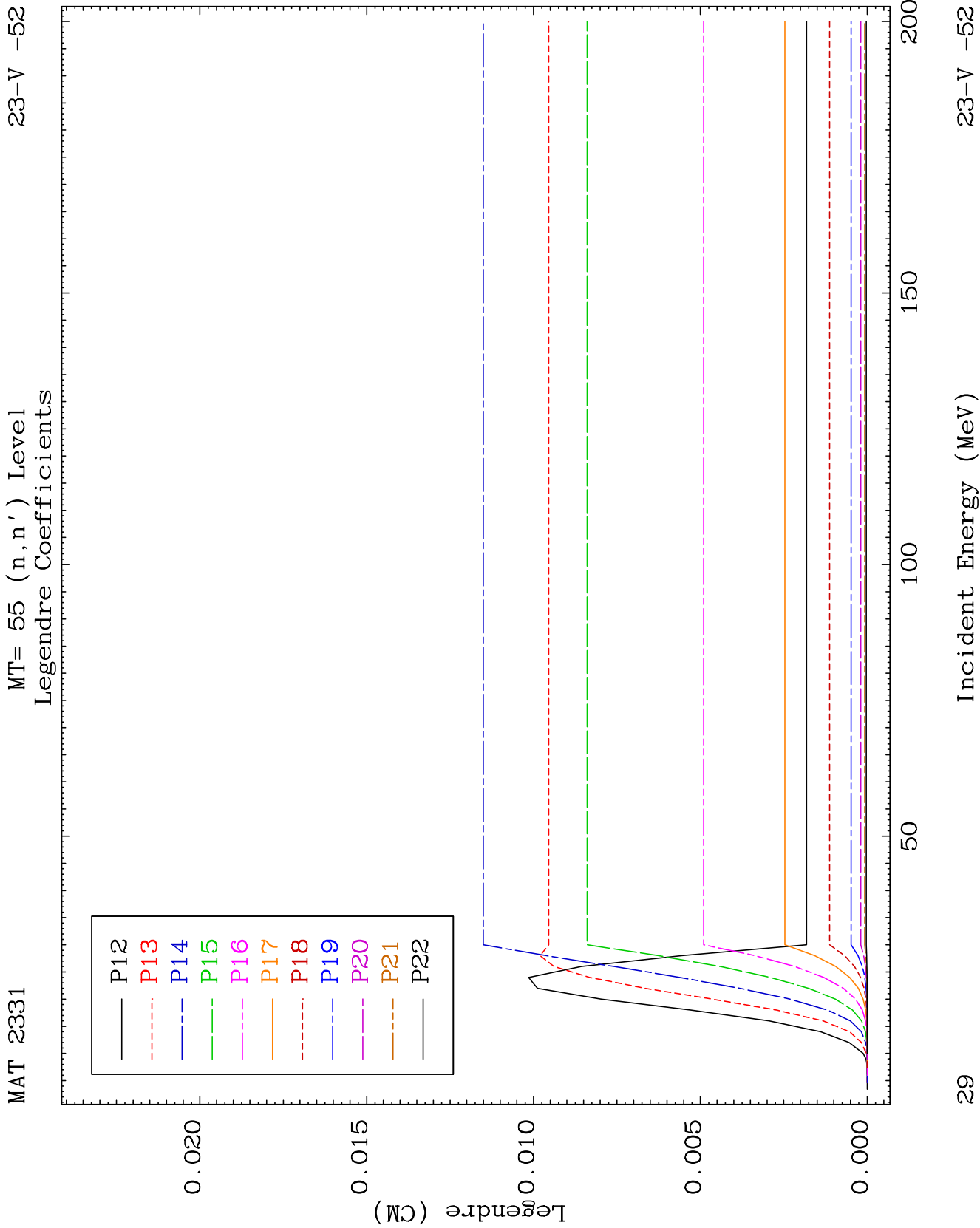


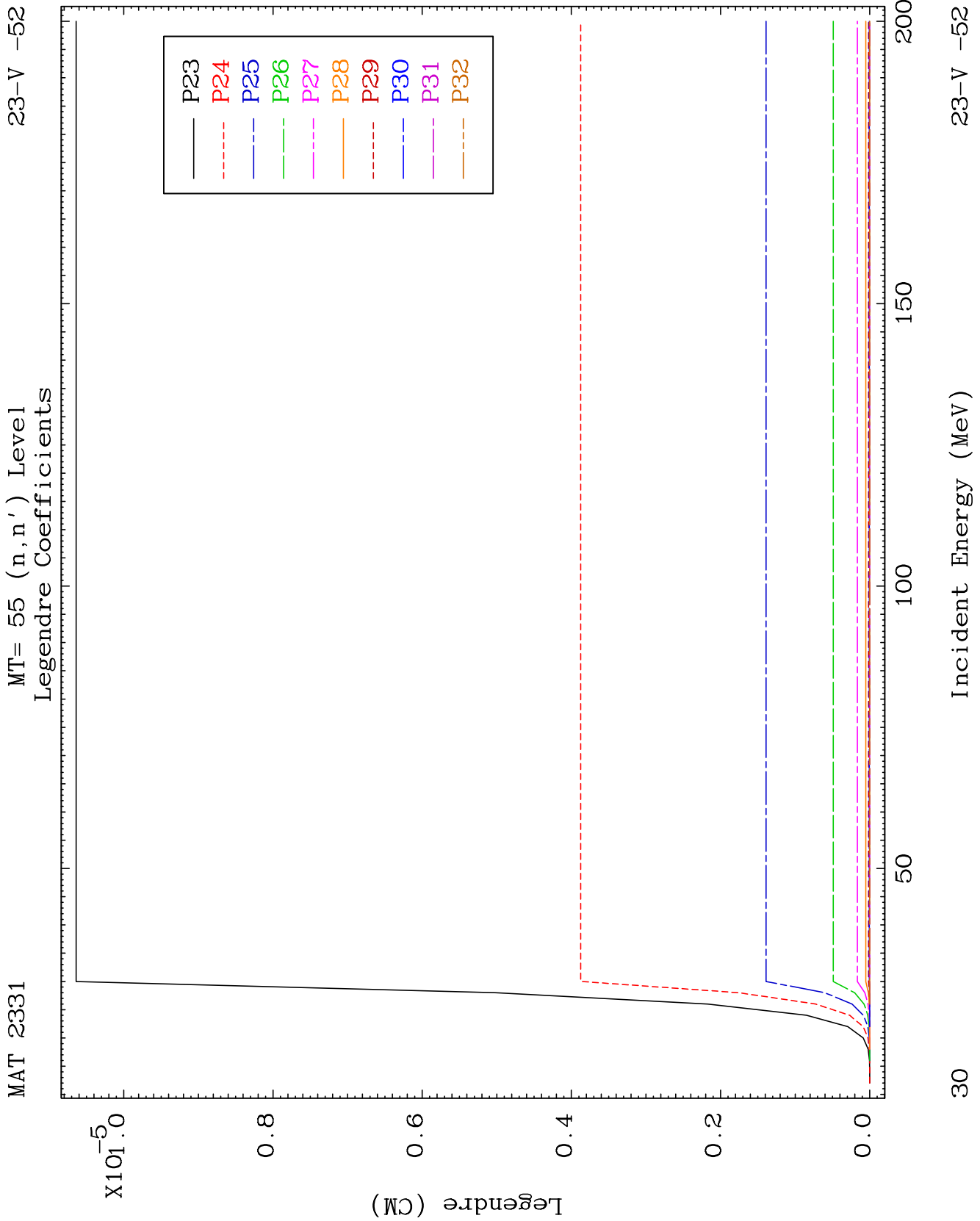








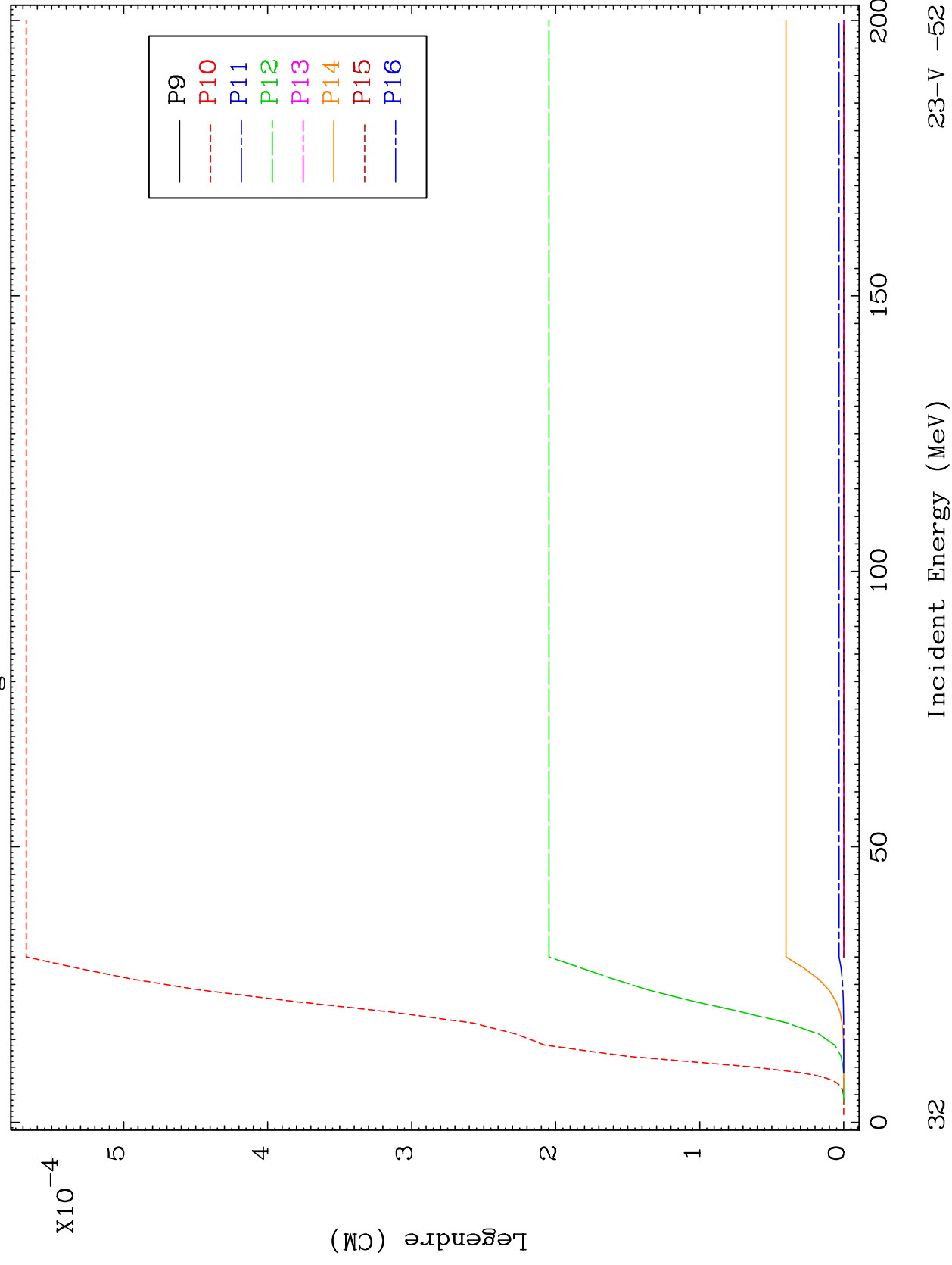


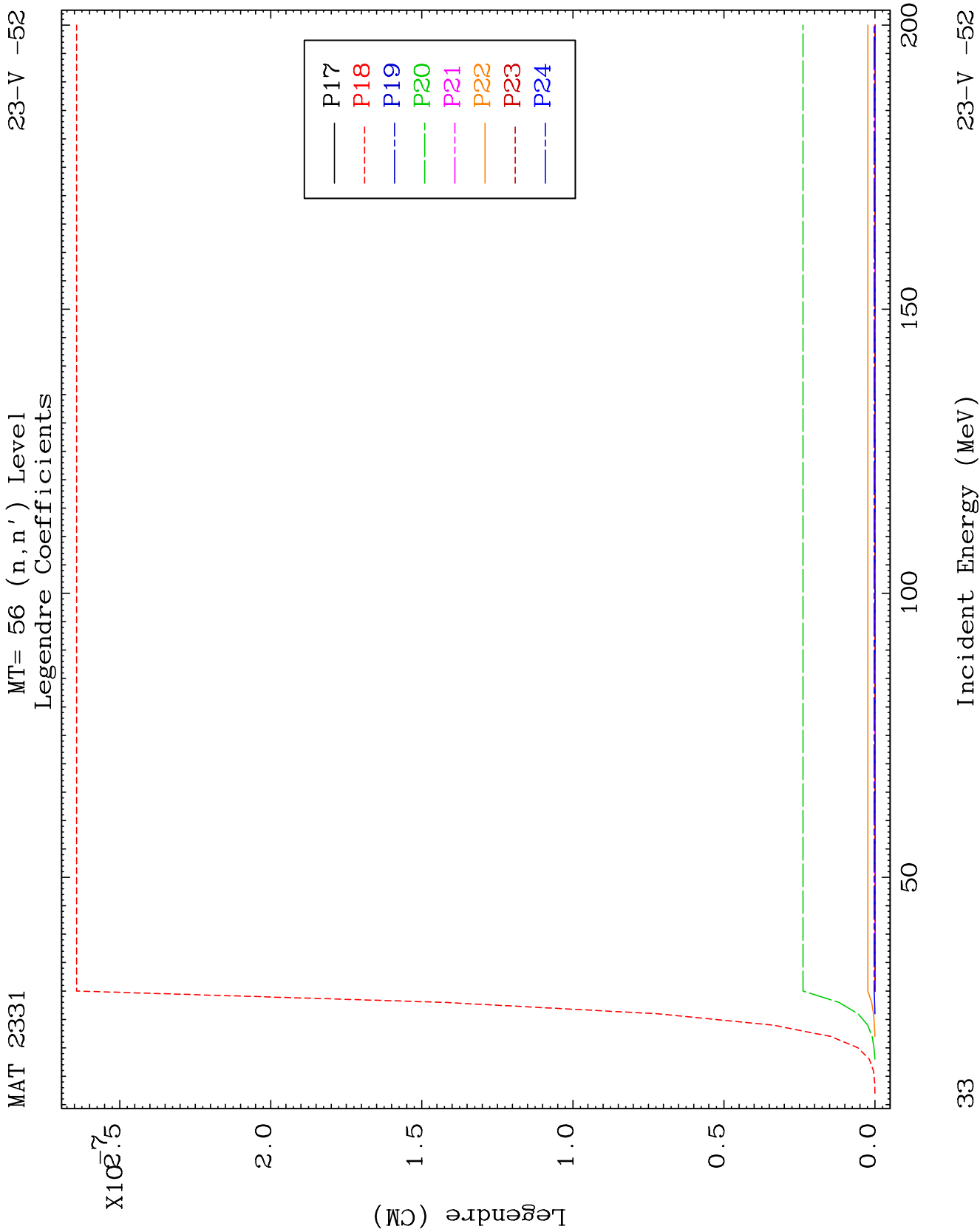


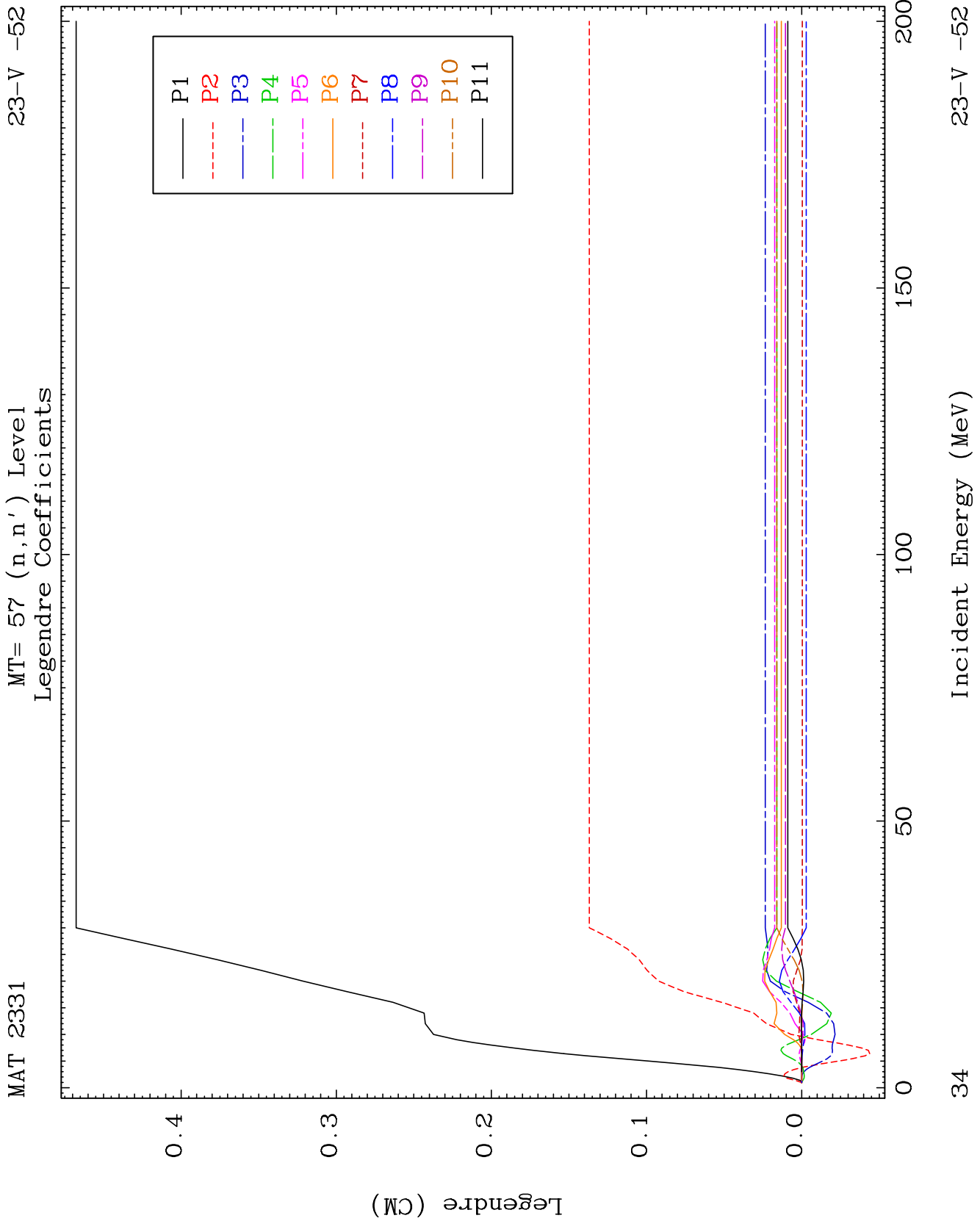
MAT 2331

MT= 56 (n, n') Level
Legendre Coefficients

23-V -52



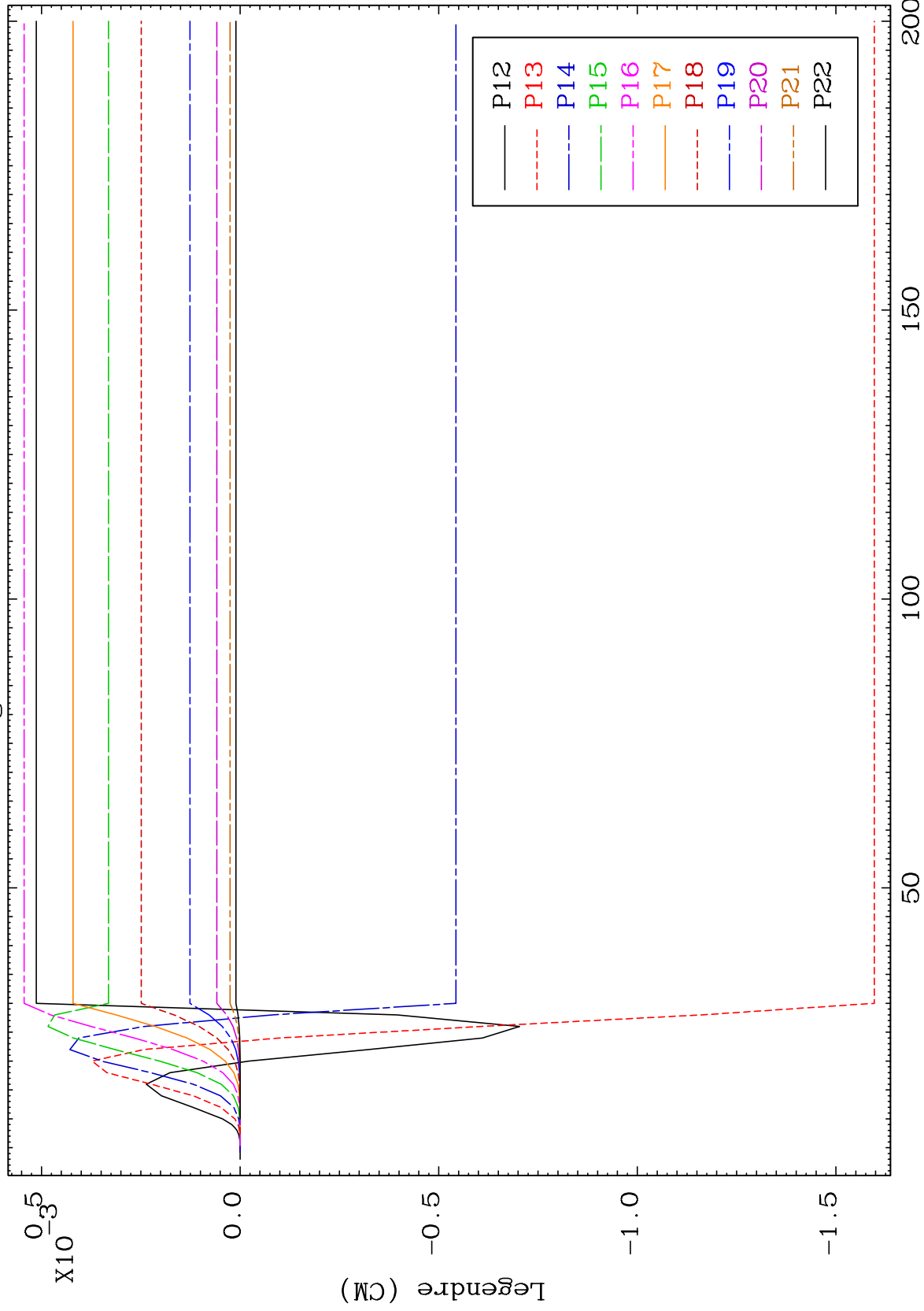




MAT 2331

MT= 57 (n,n') Level
Legendre Coefficients

23-V -52



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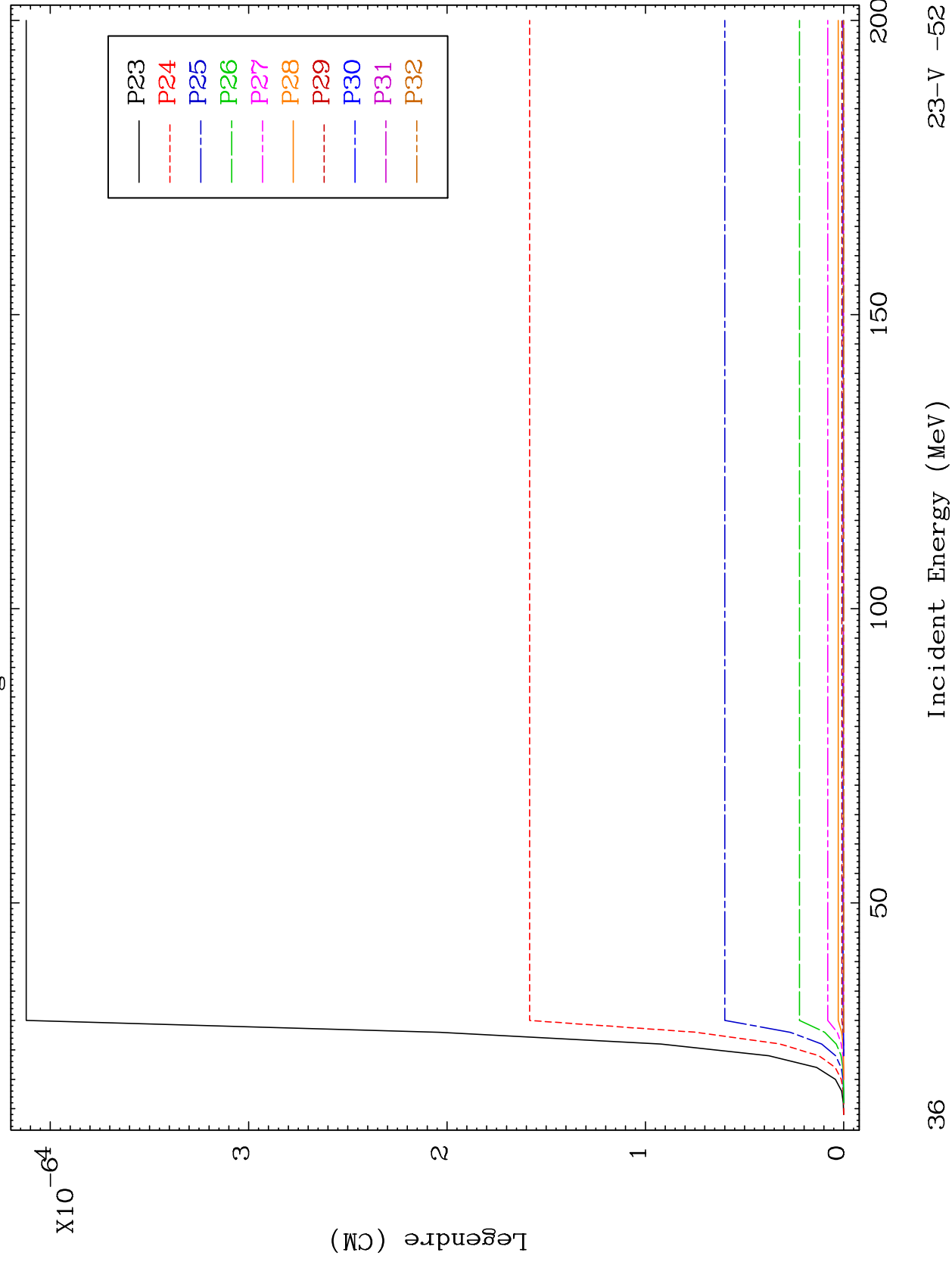
Incident Energy (MeV)

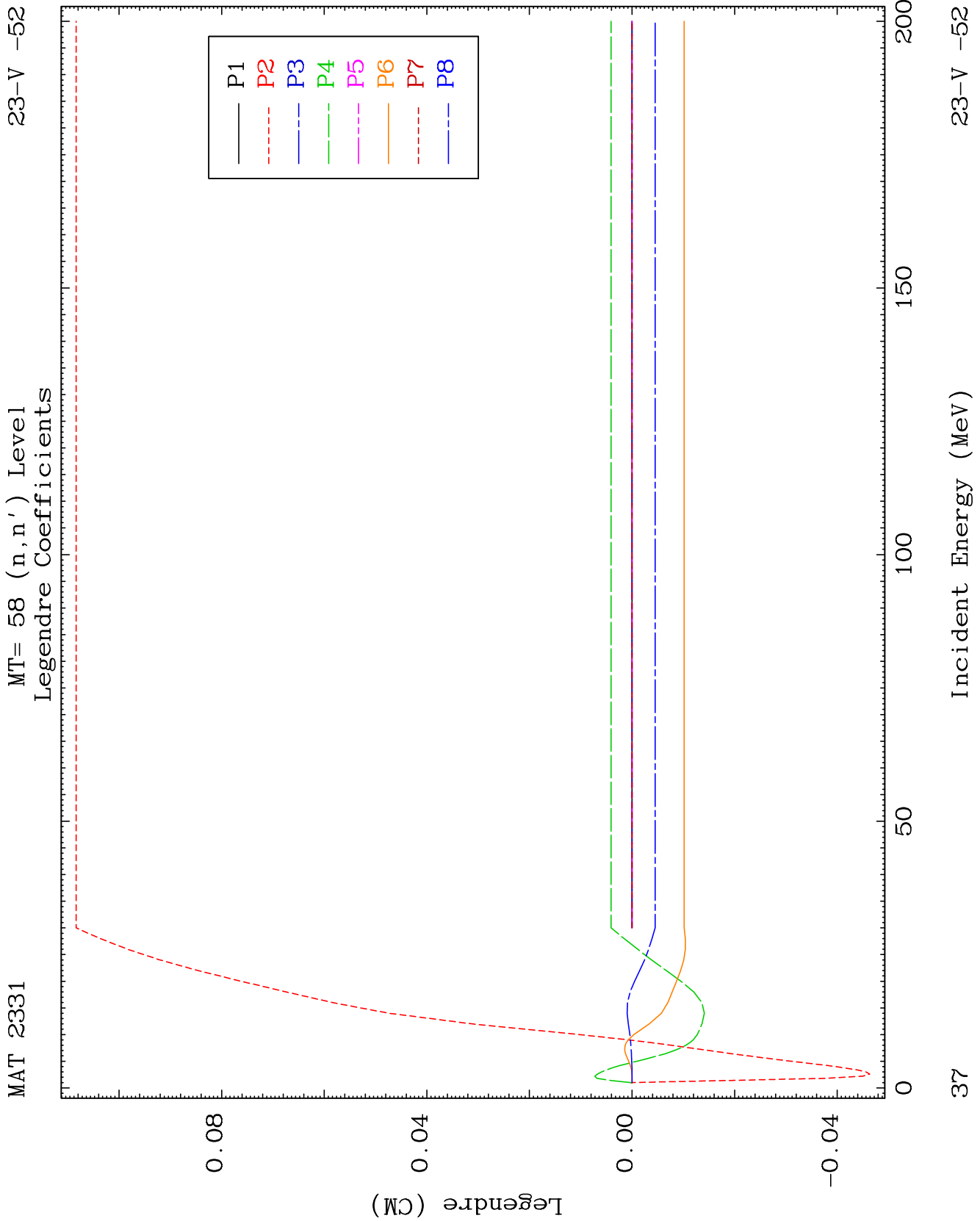
23-V -52

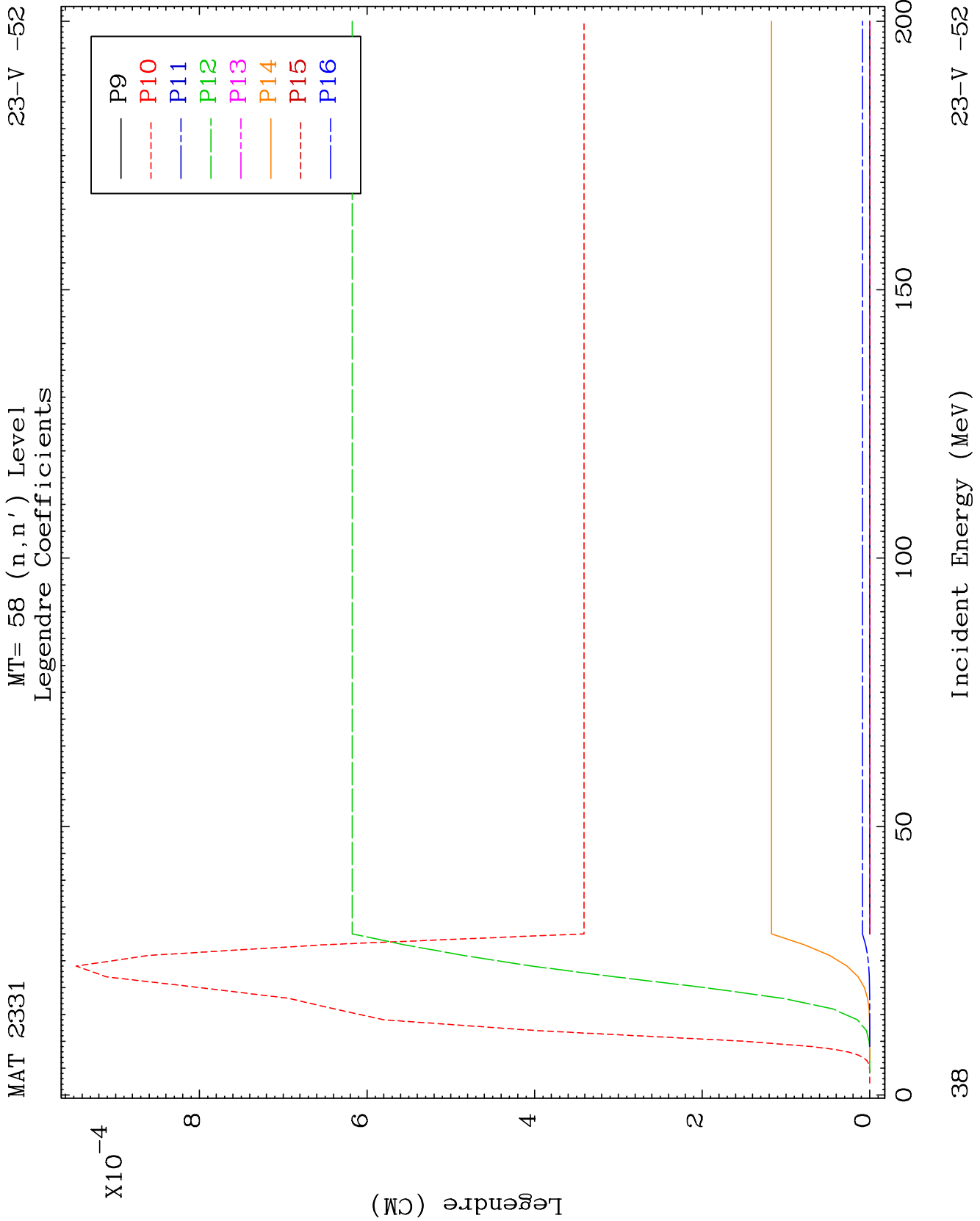
MAT 2331

MT= 57 (n, n') Level
Legendre Coefficients

23-V -52



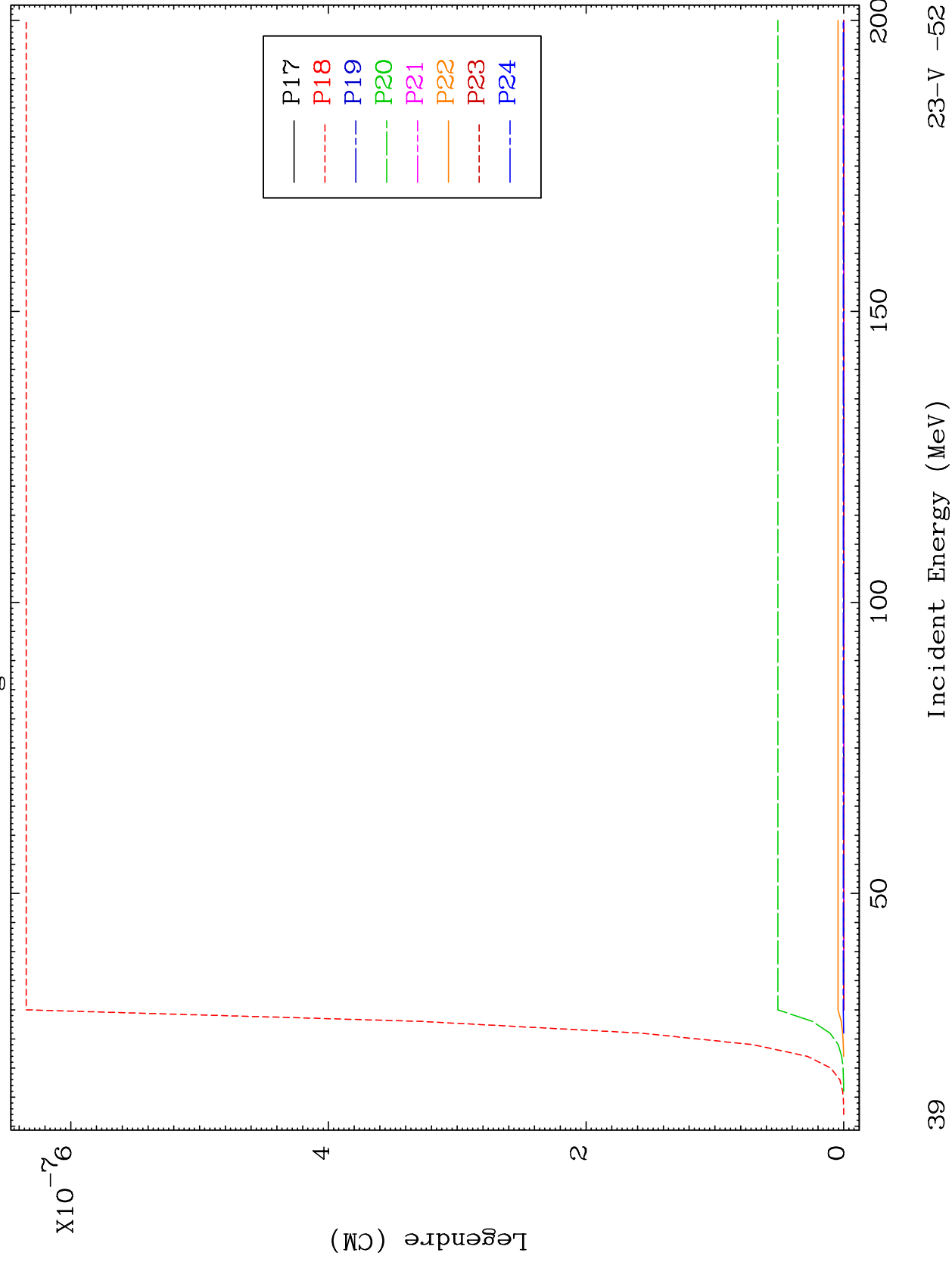




MAT 2331

MT= 58 (n, n') Level
Legendre Coefficients

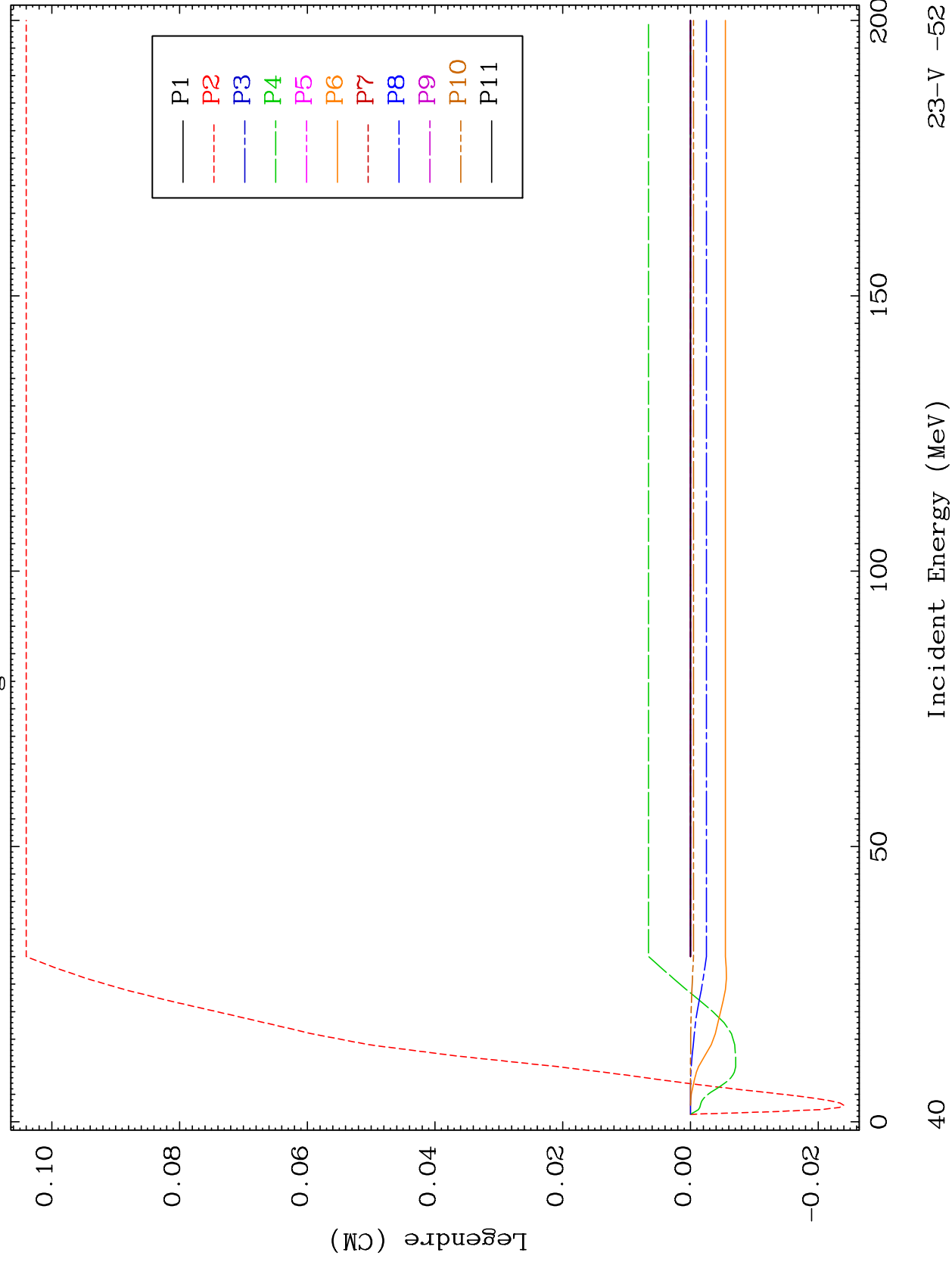
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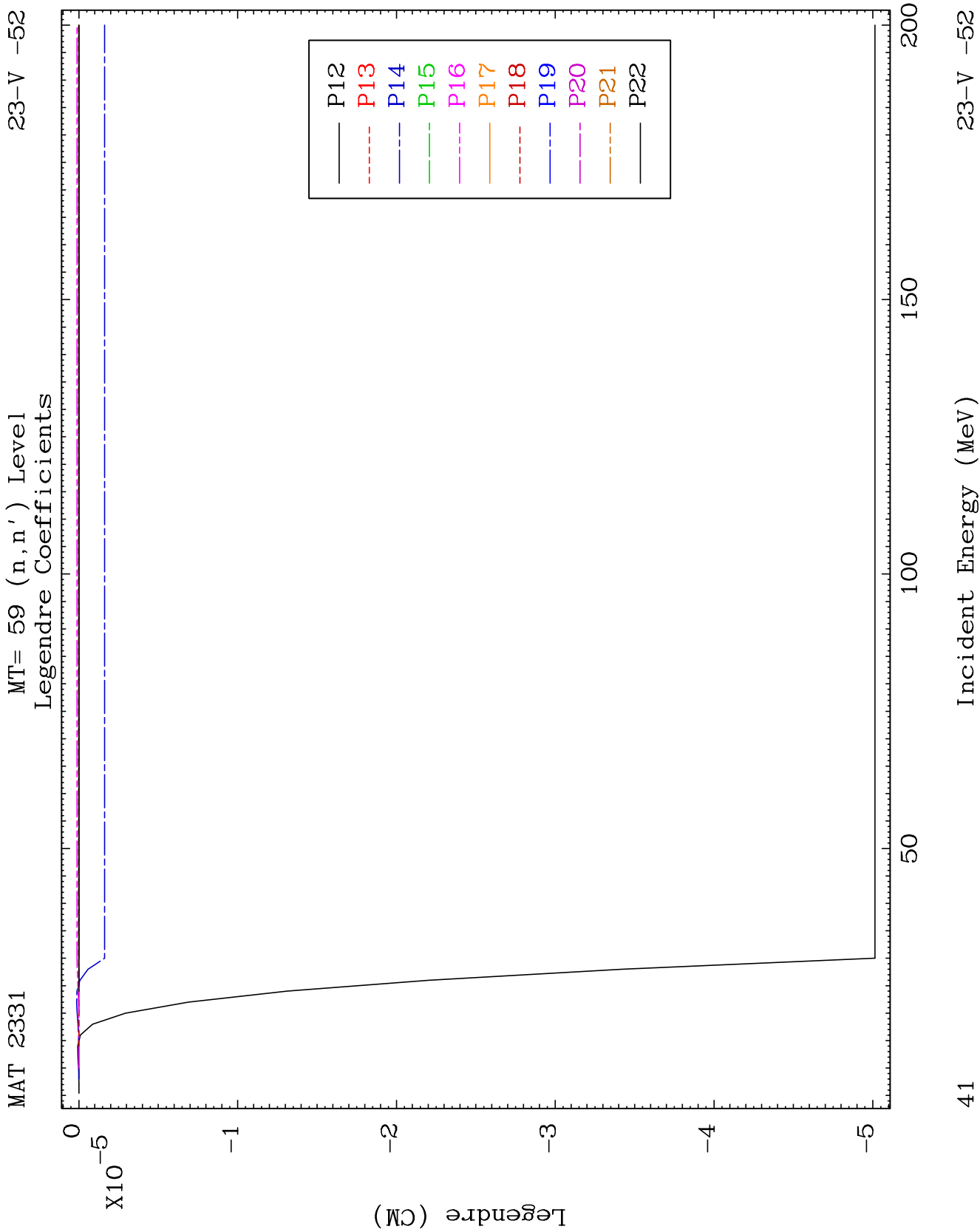


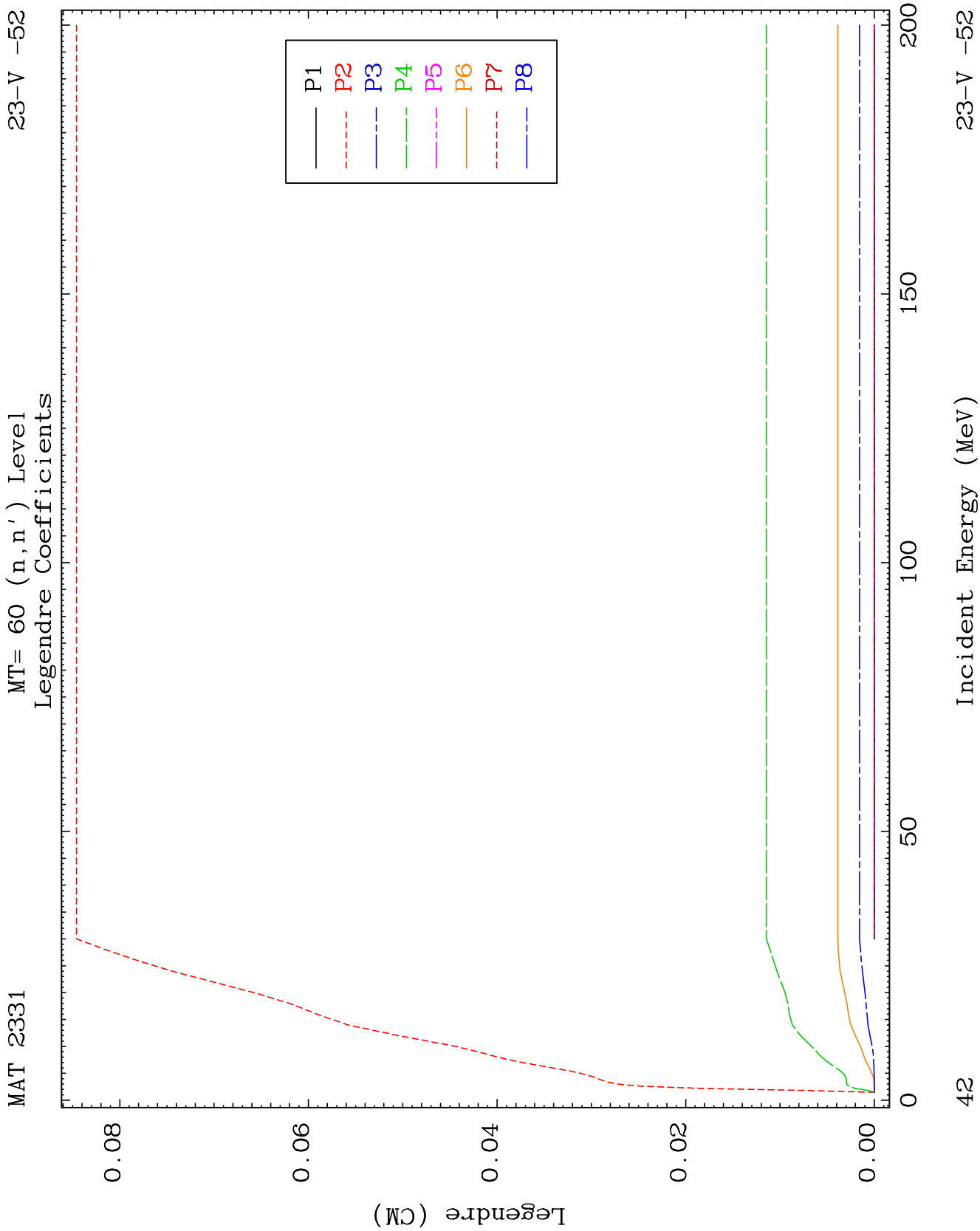
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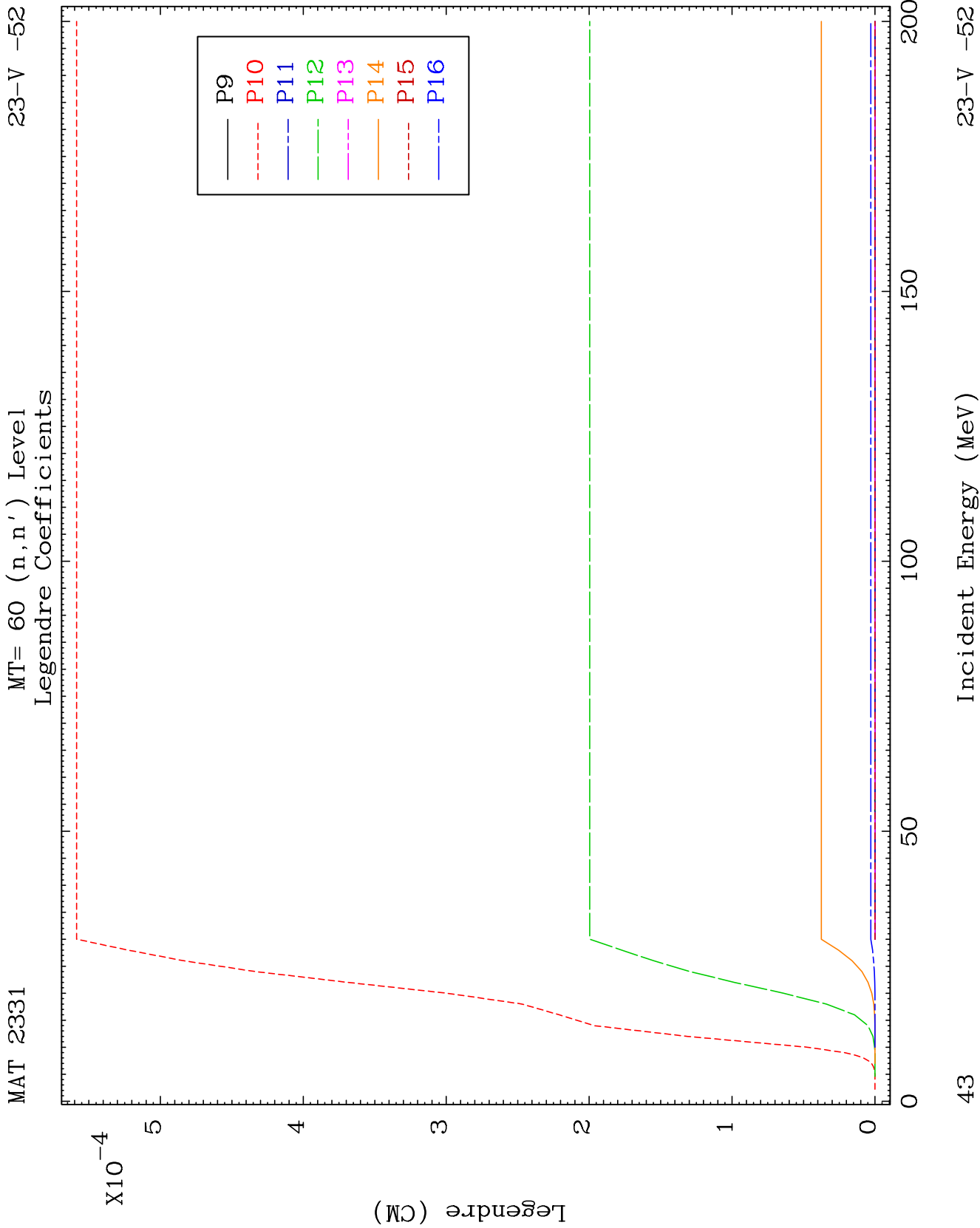
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Legendre Coefficients

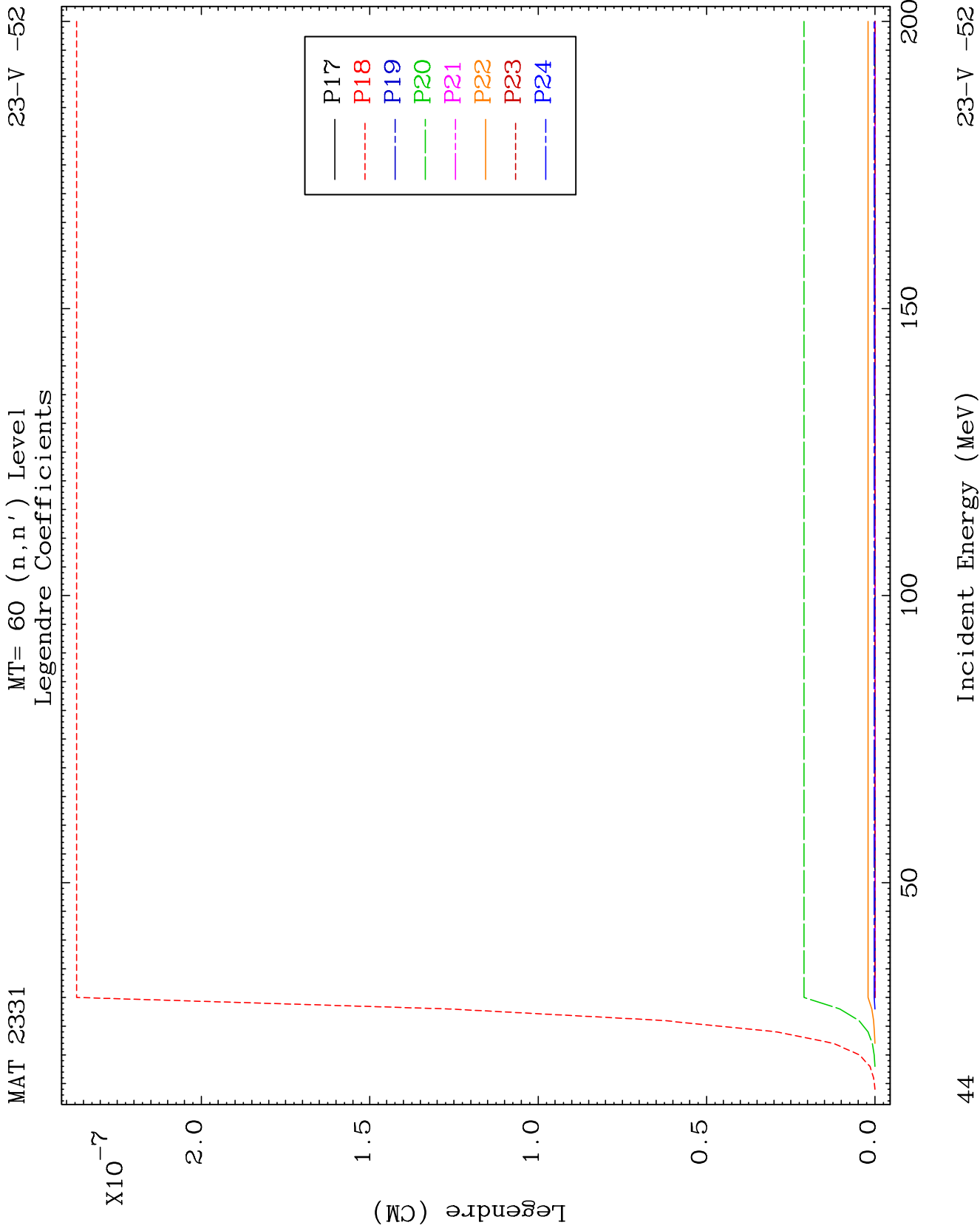
23-V -52







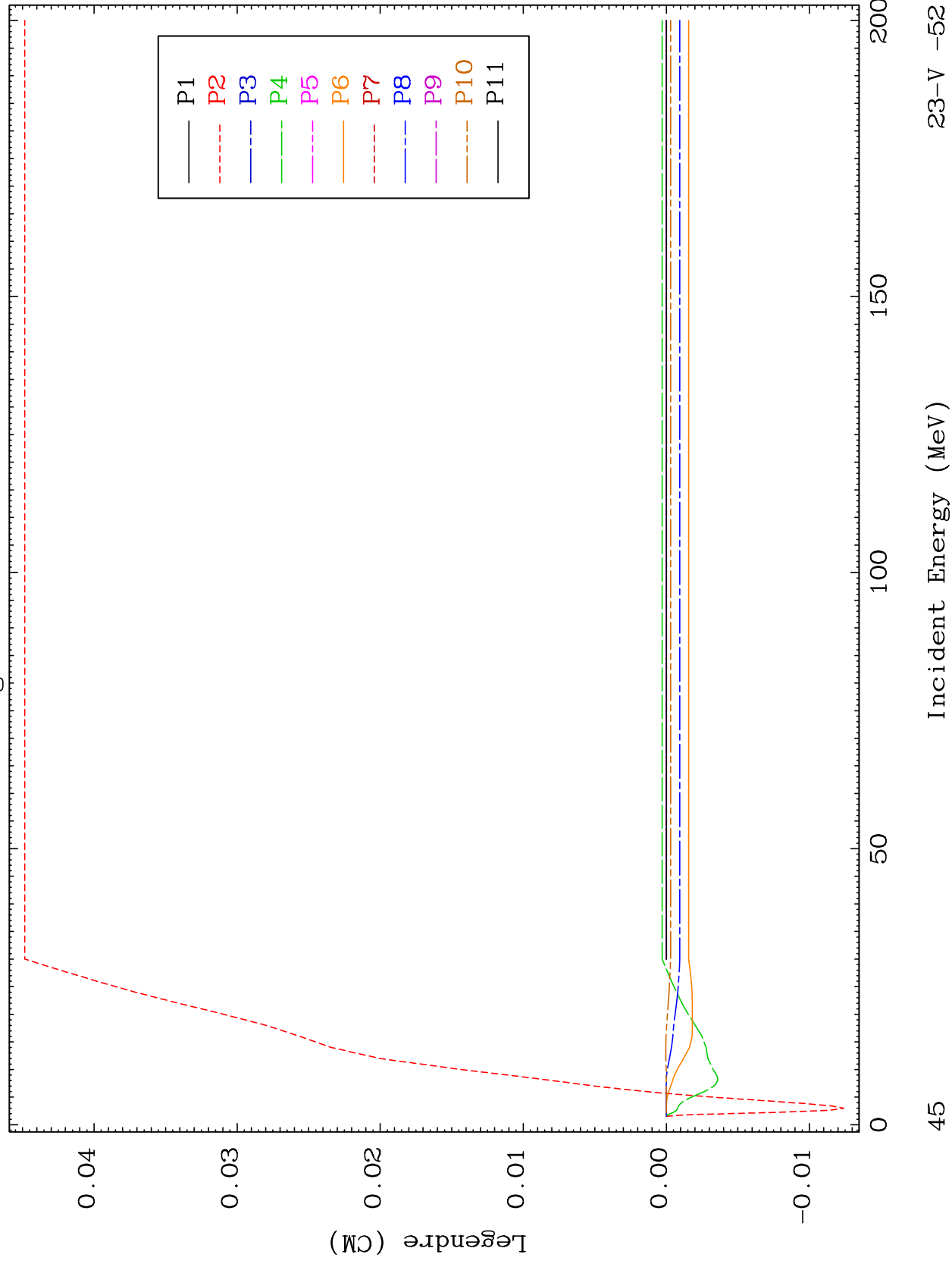


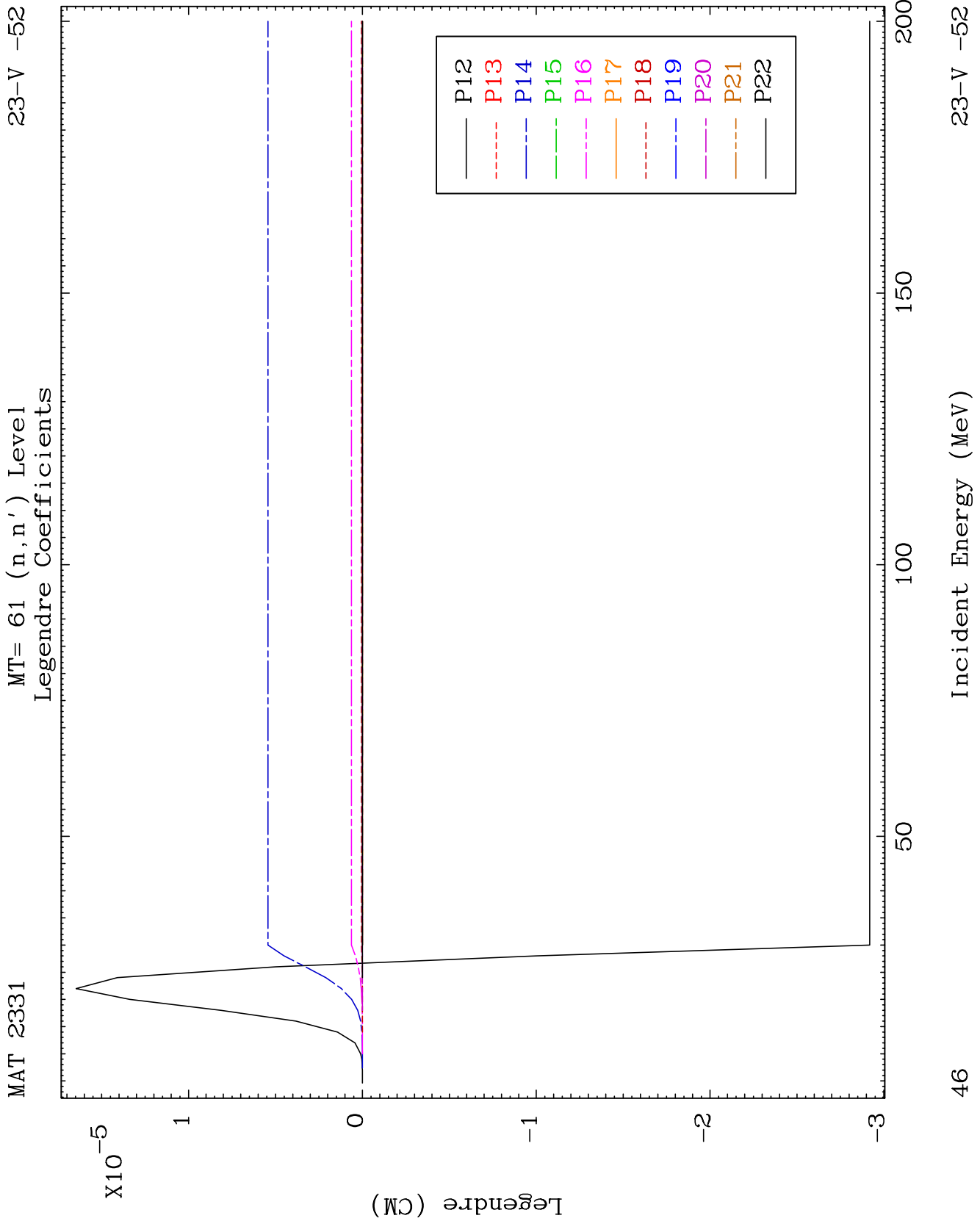


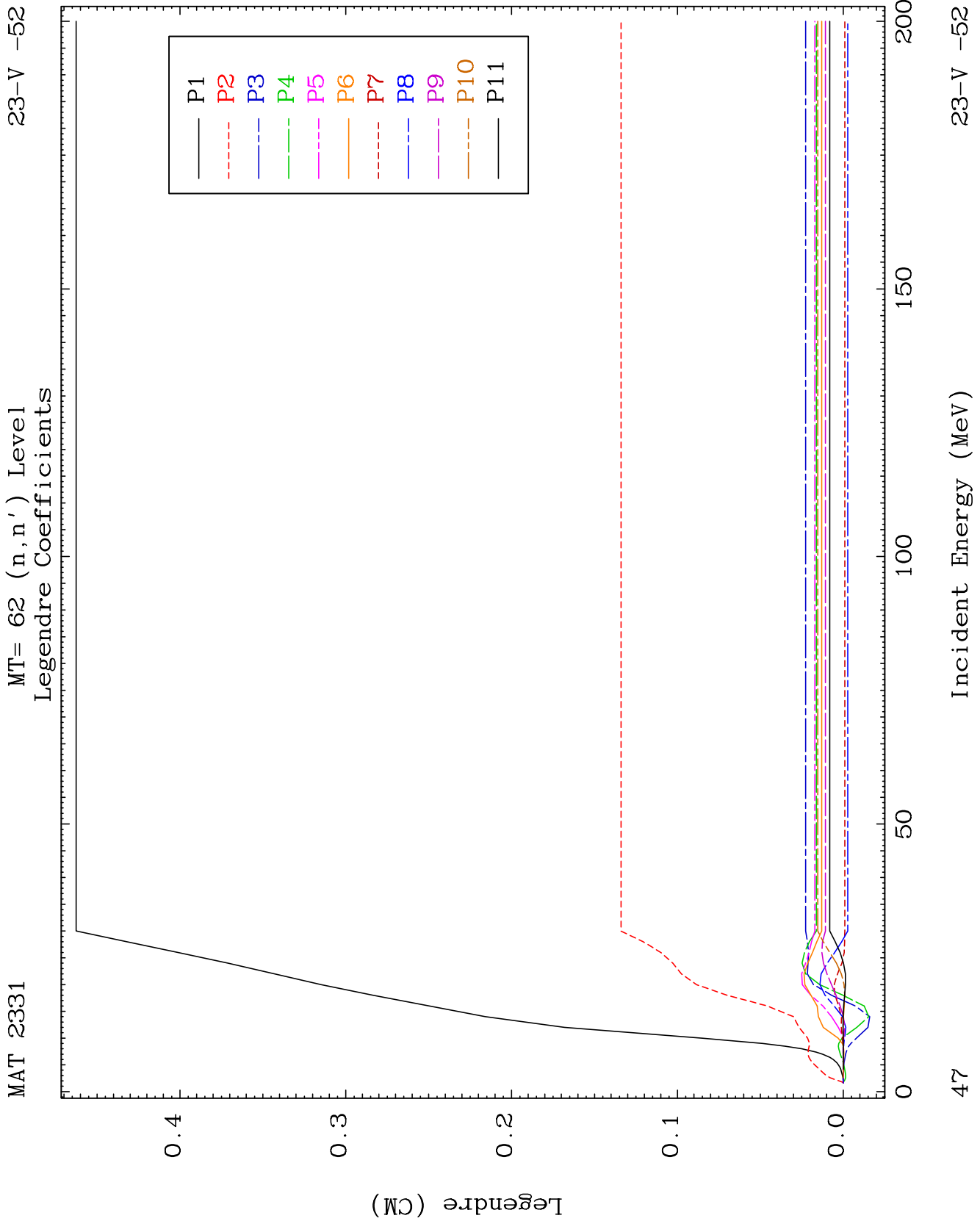
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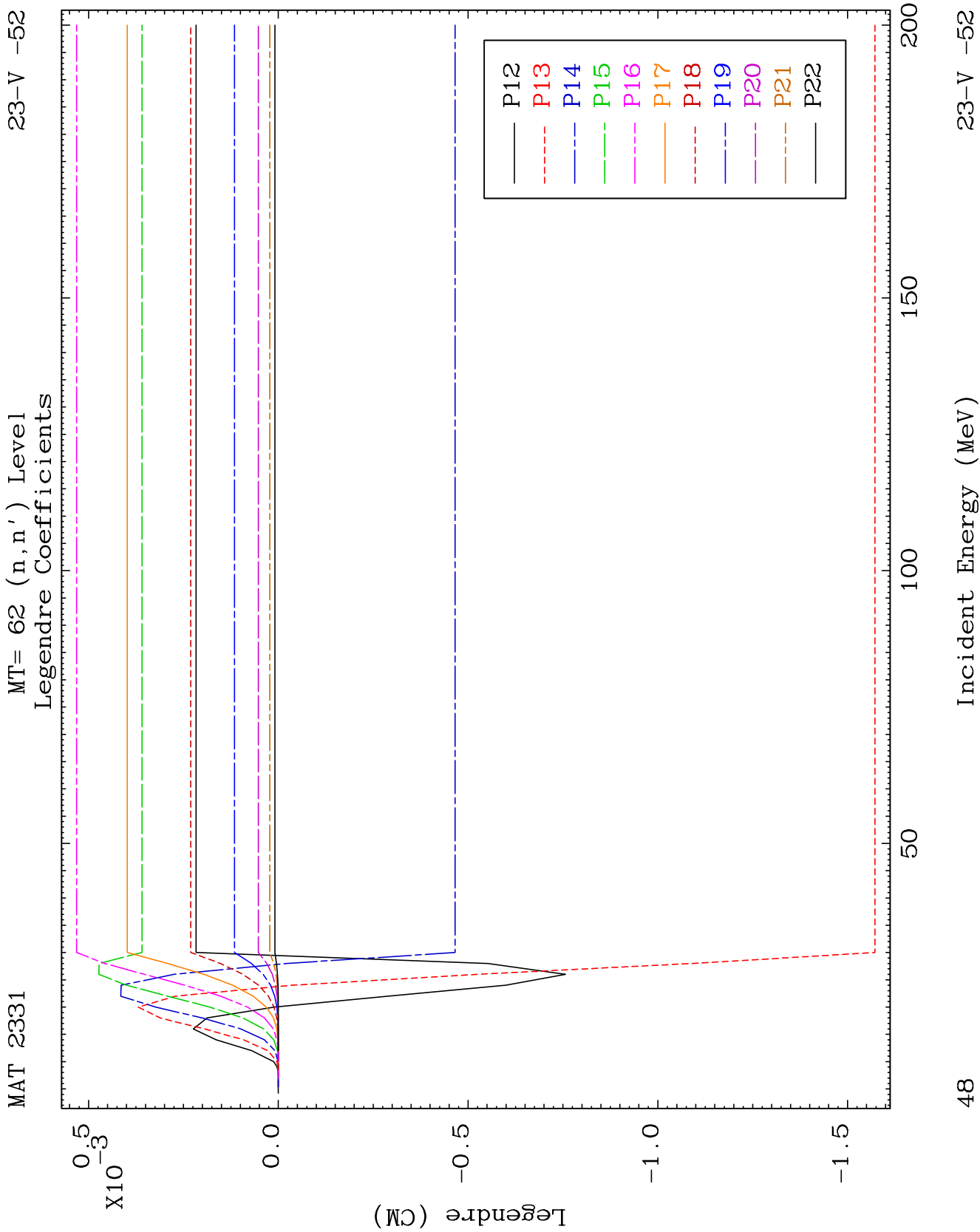
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Legendre Coefficients

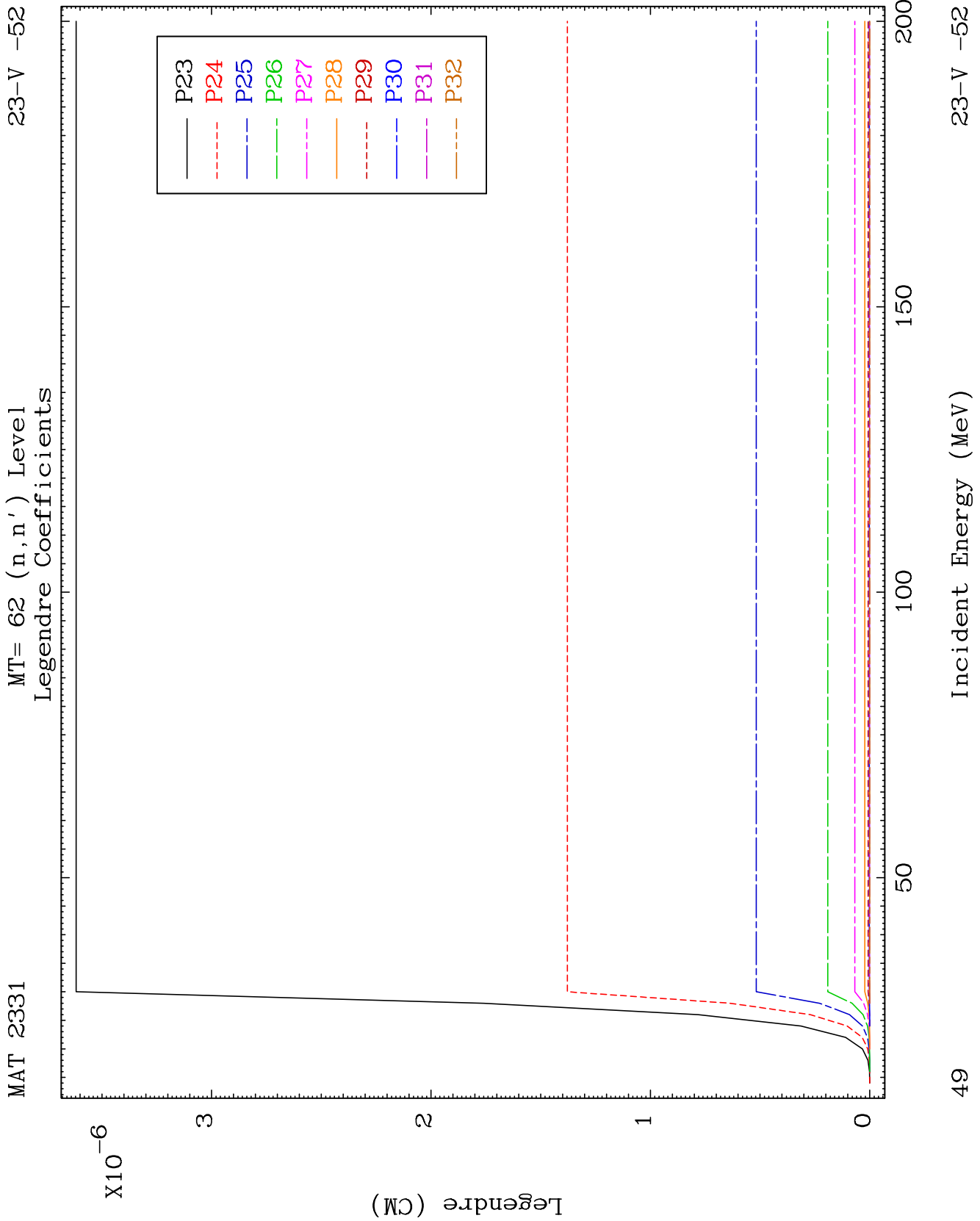
23-V -52







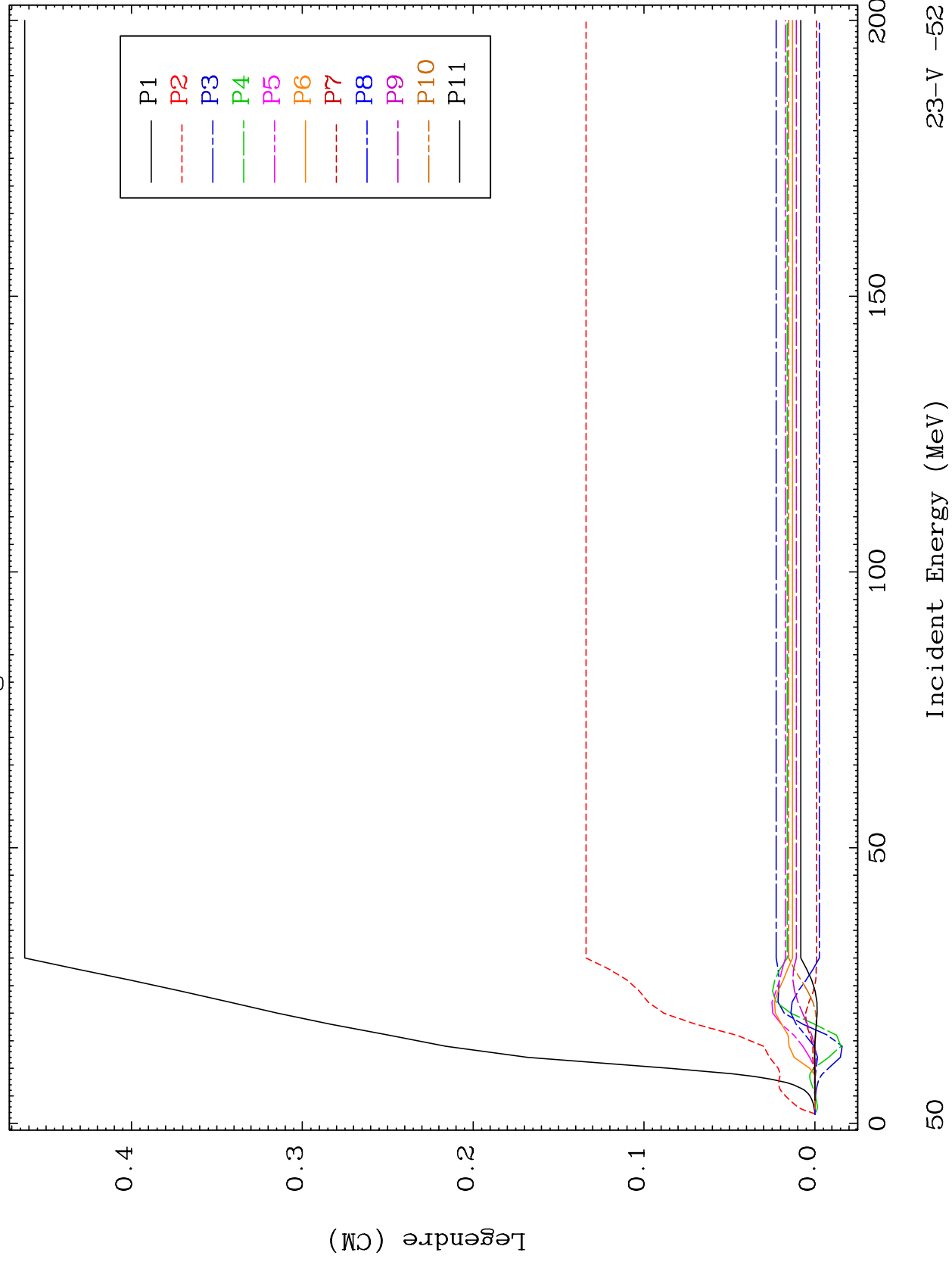


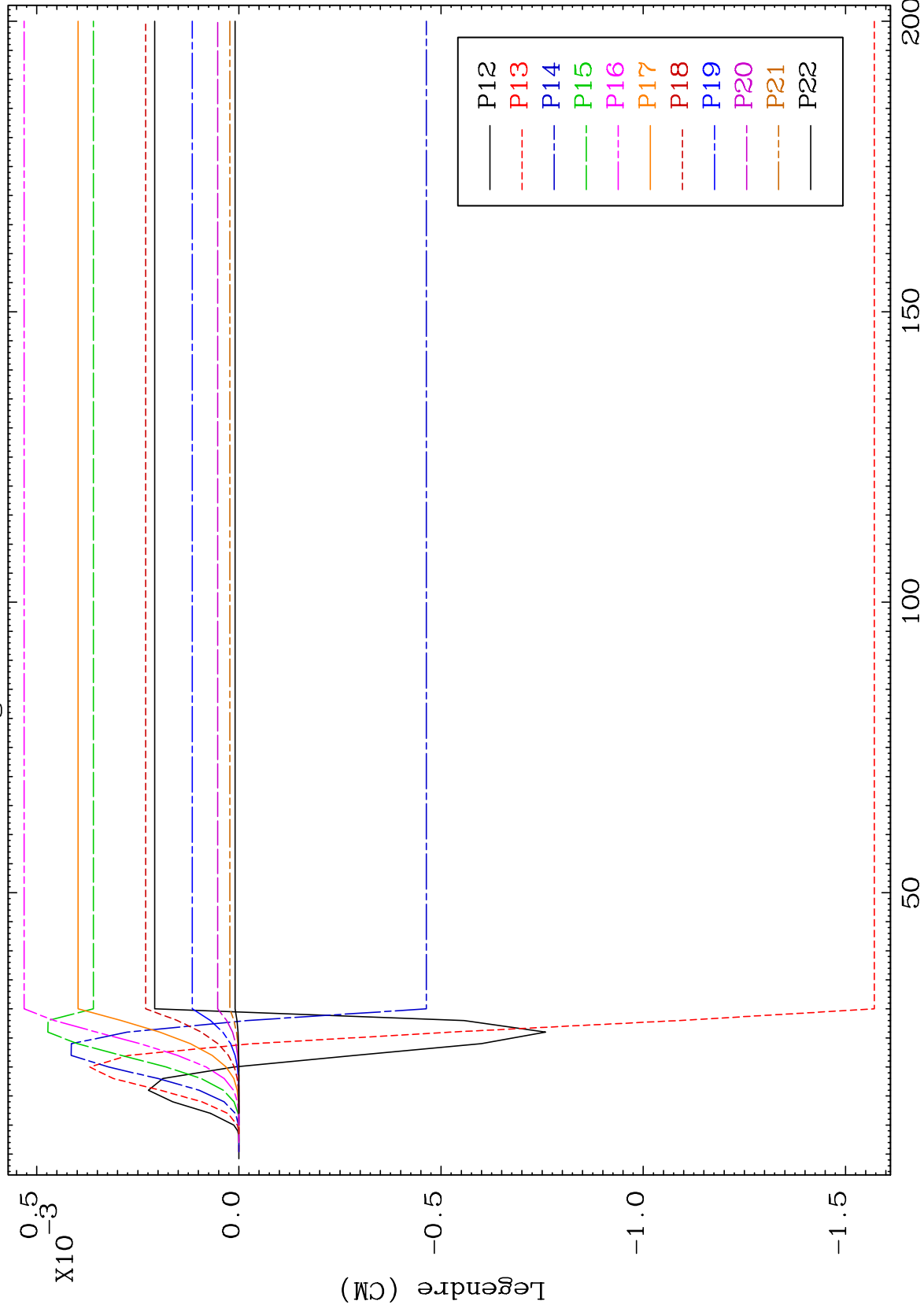


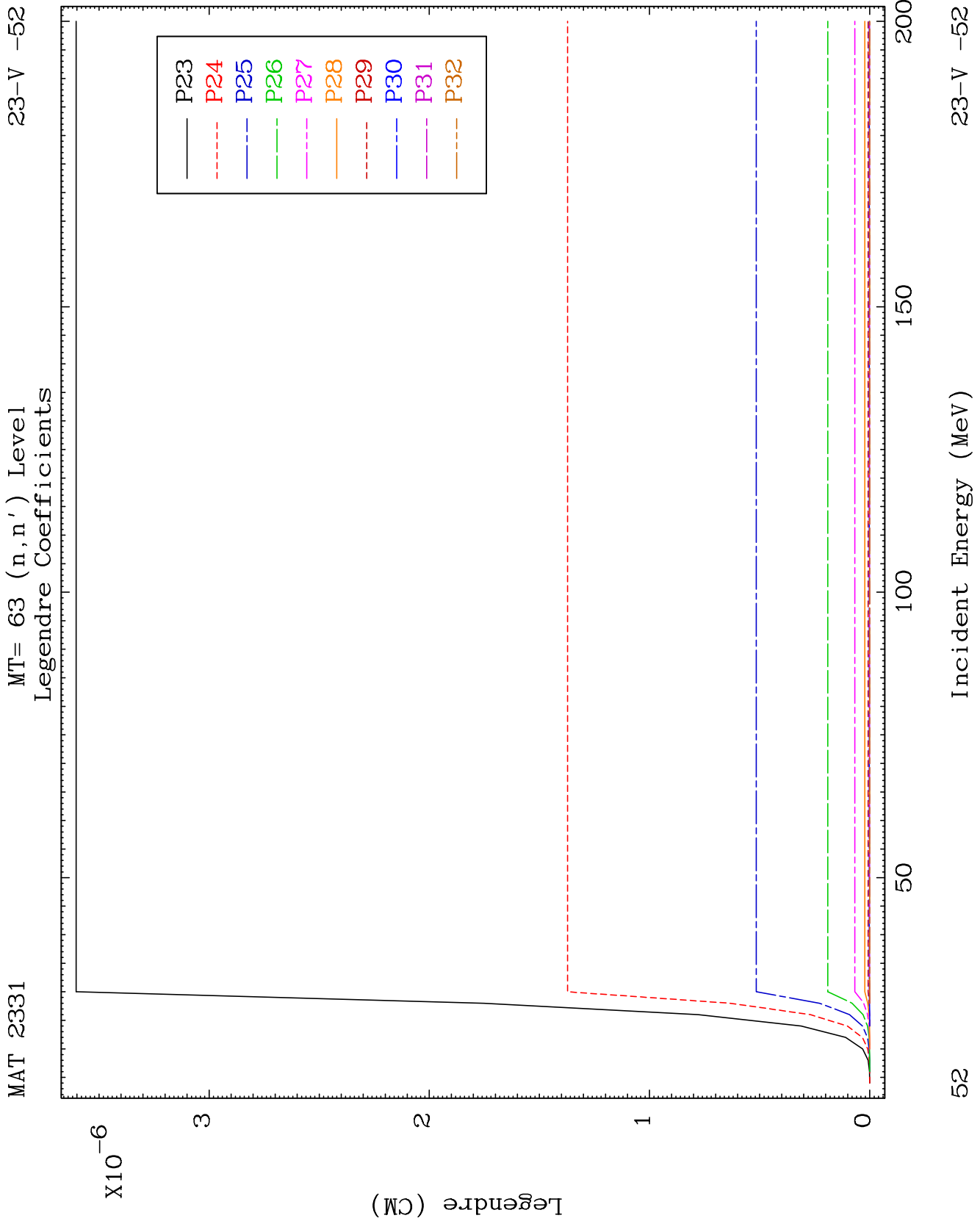
MAT 2331

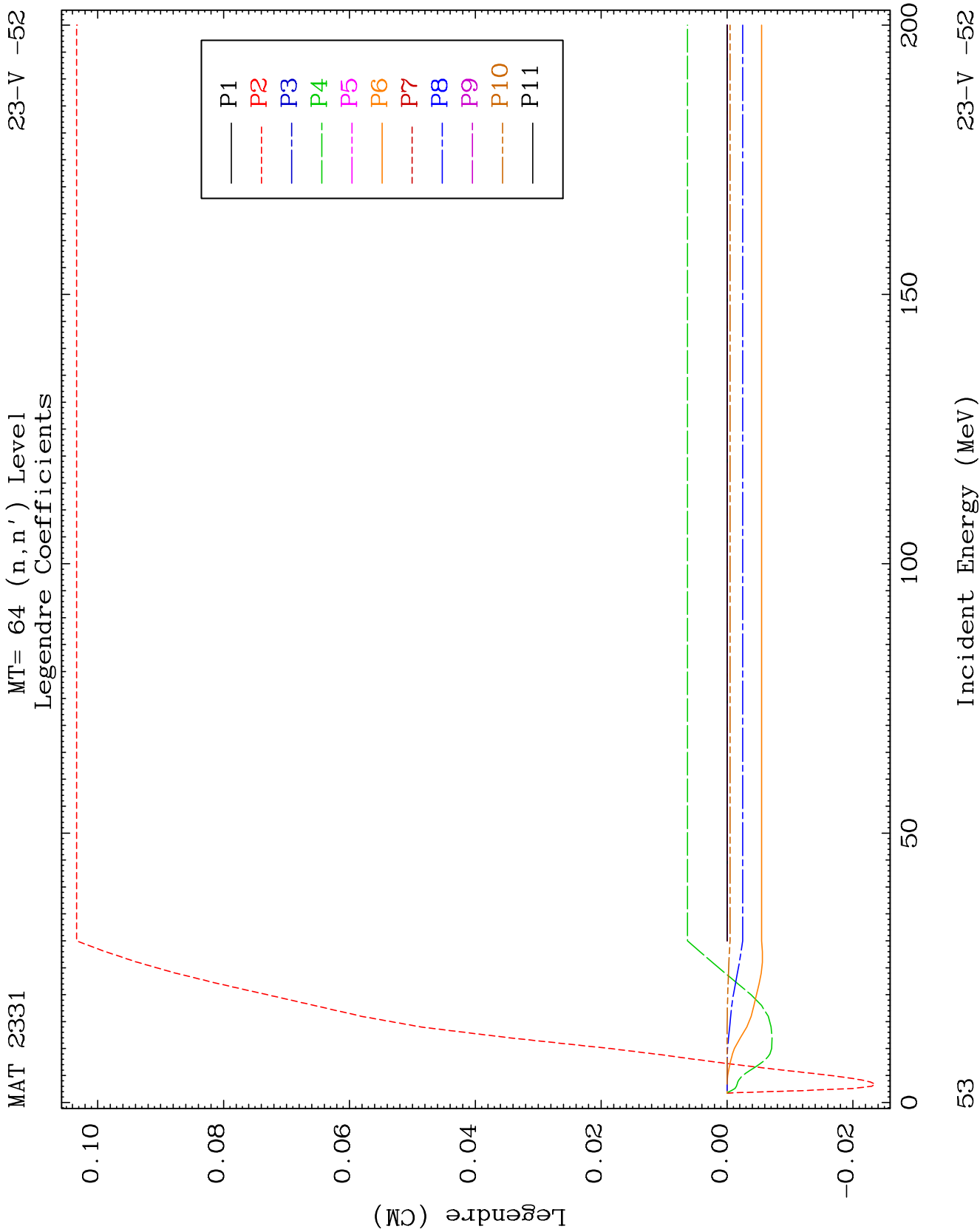
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Legendre Coefficients

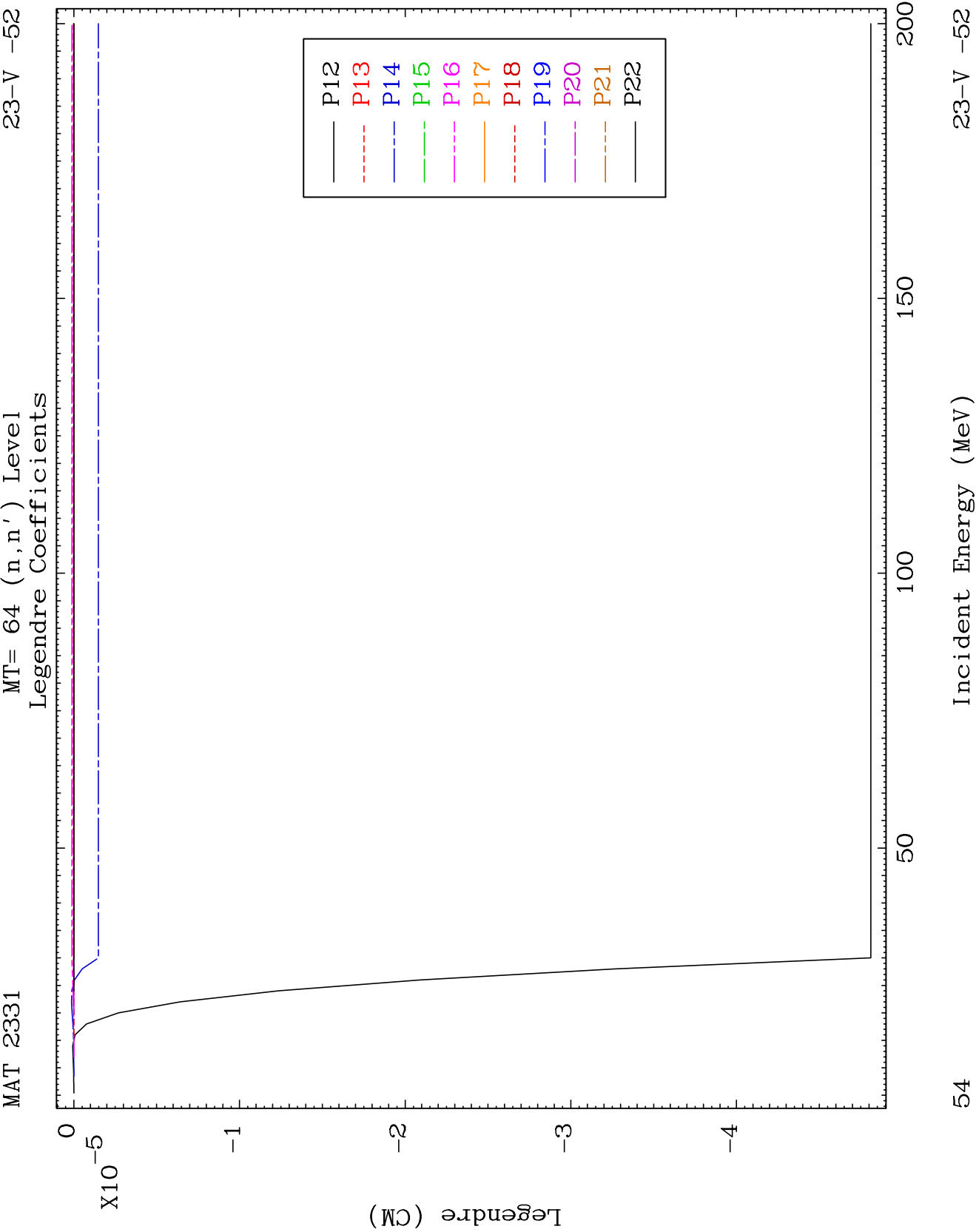
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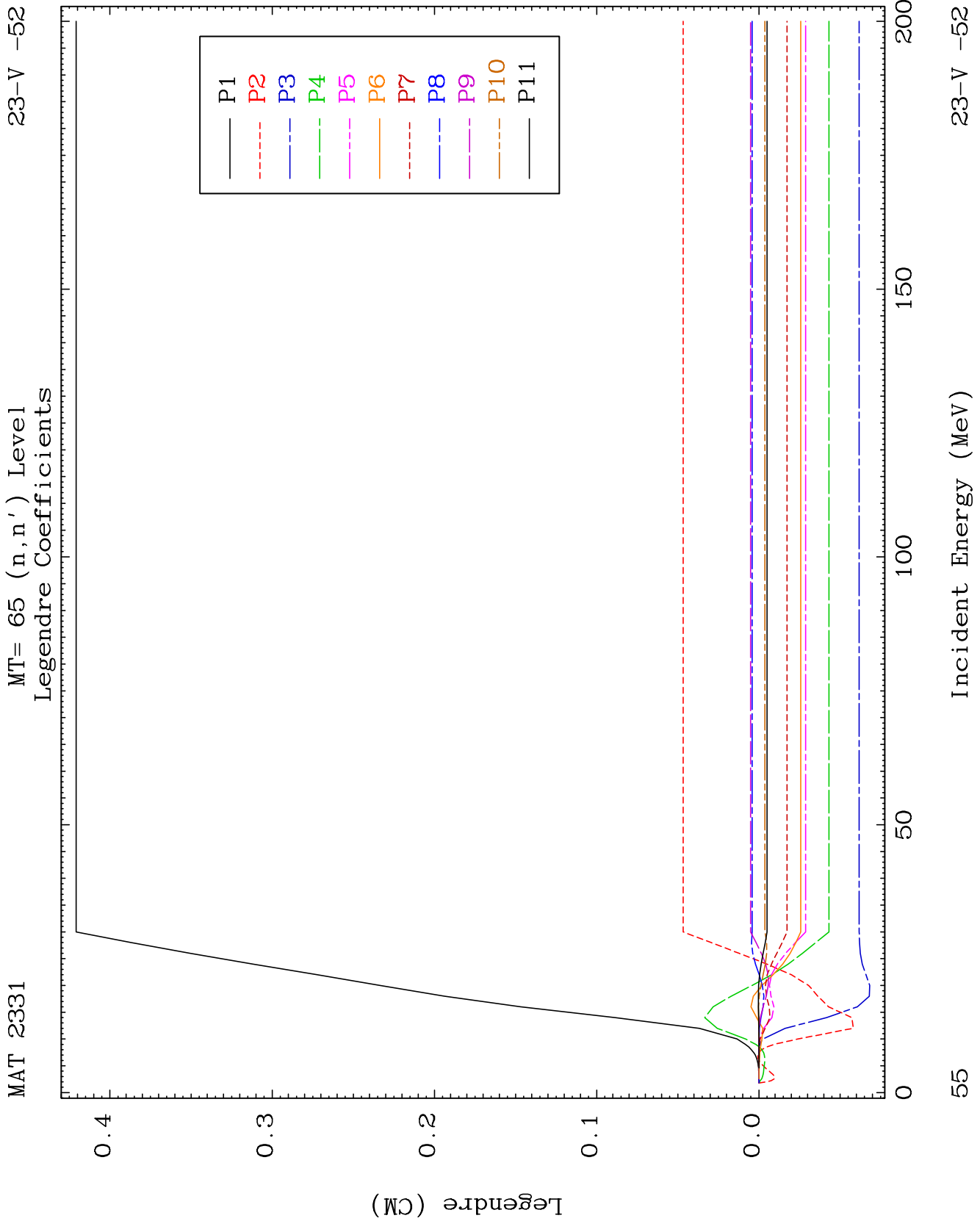


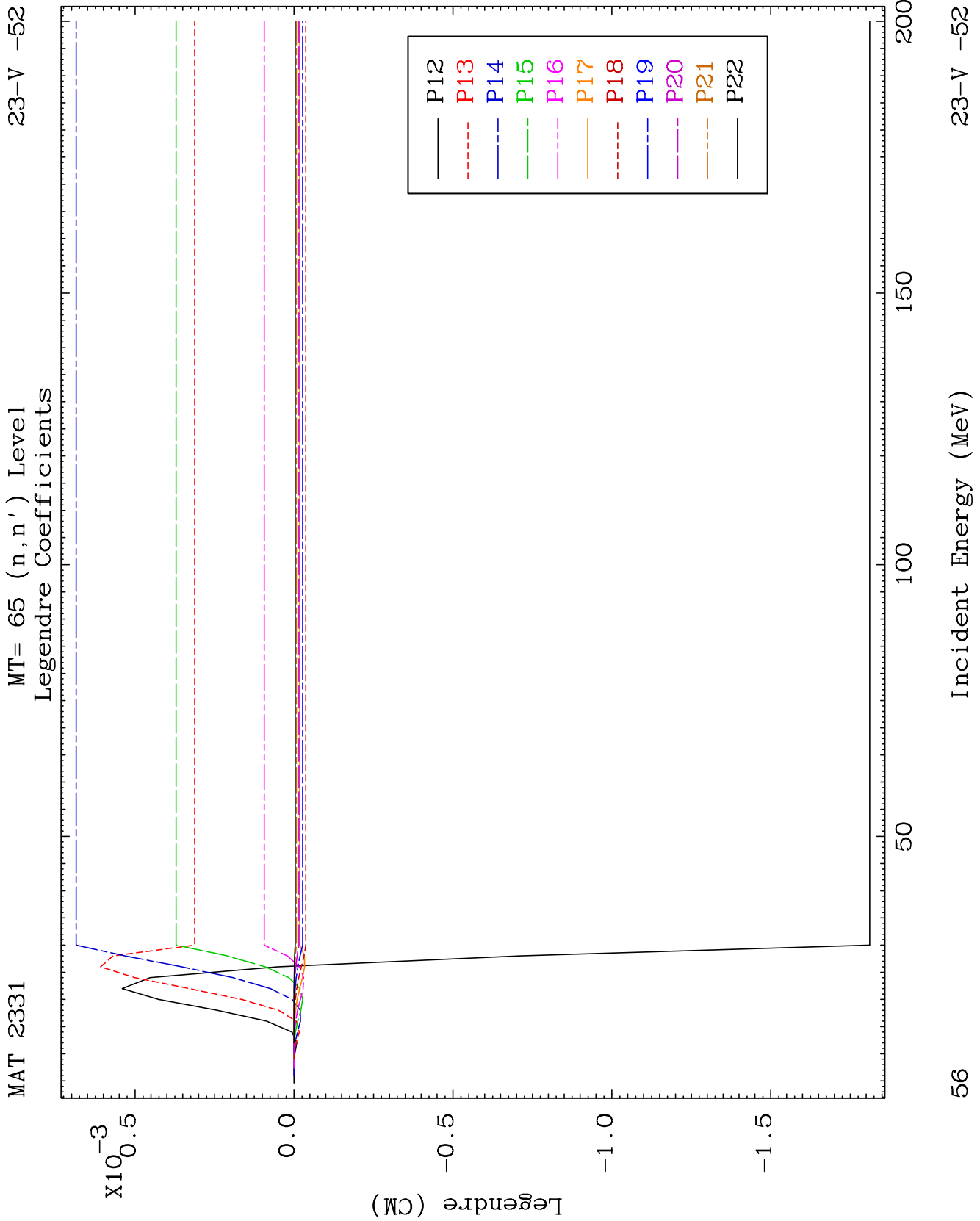


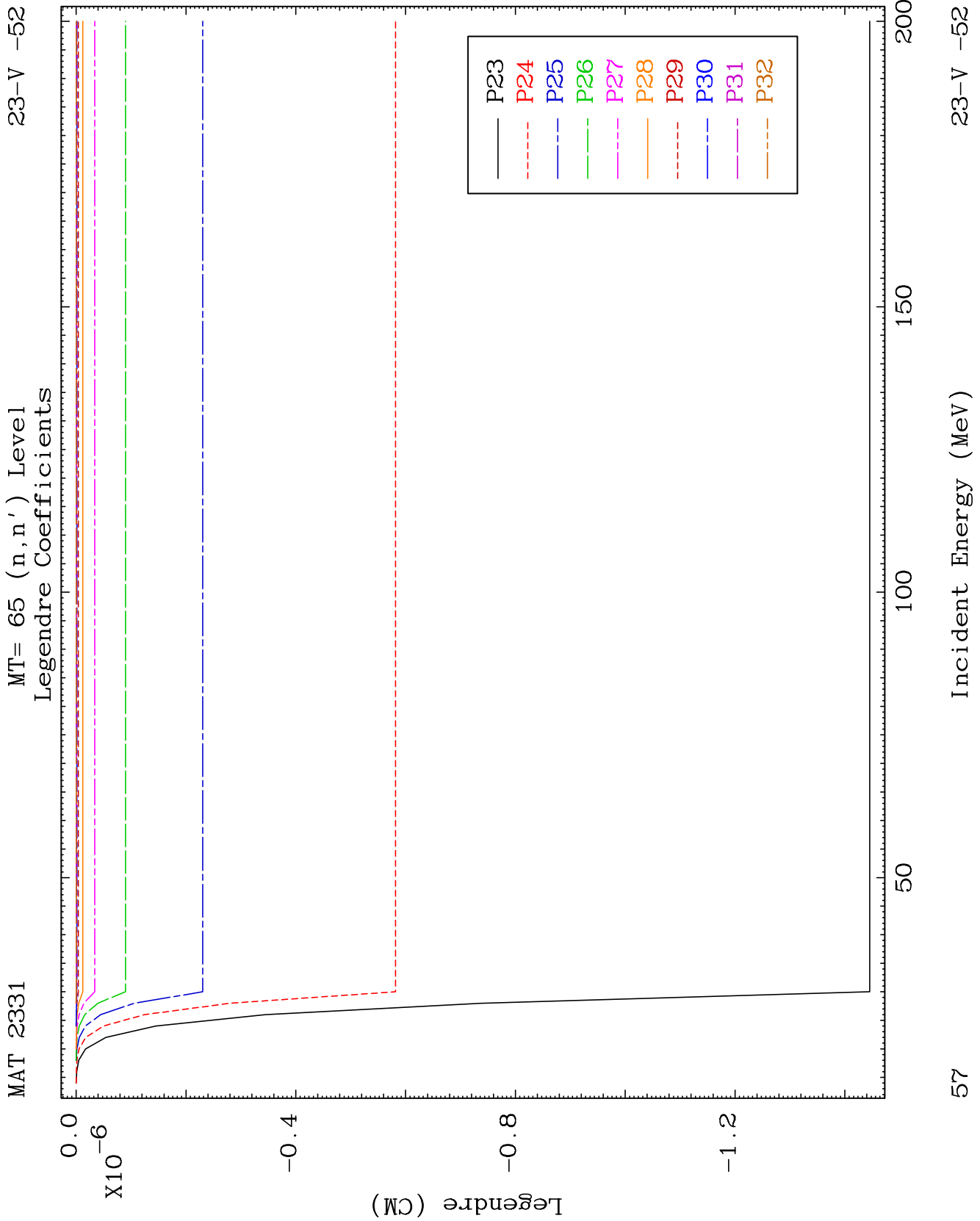




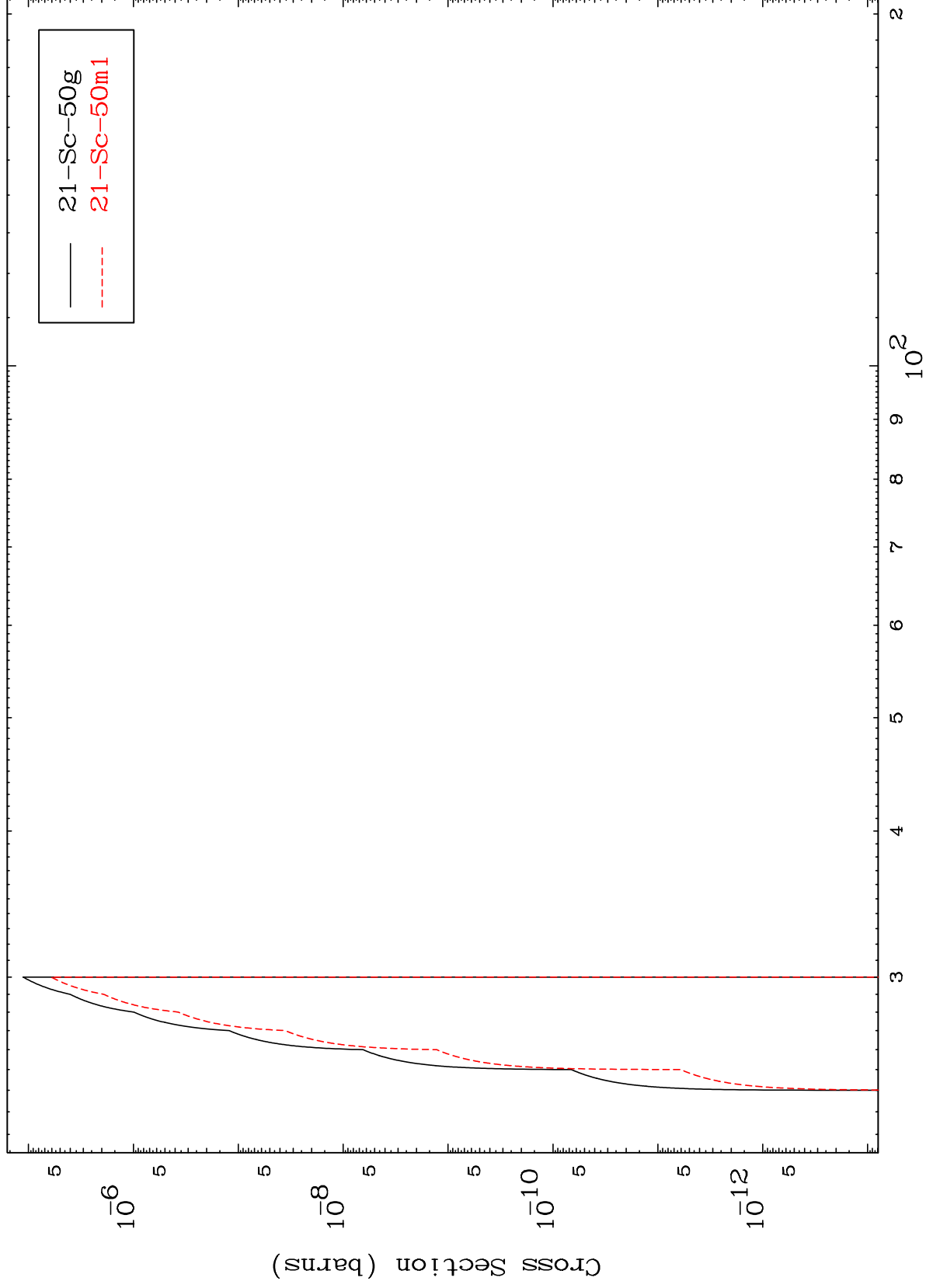




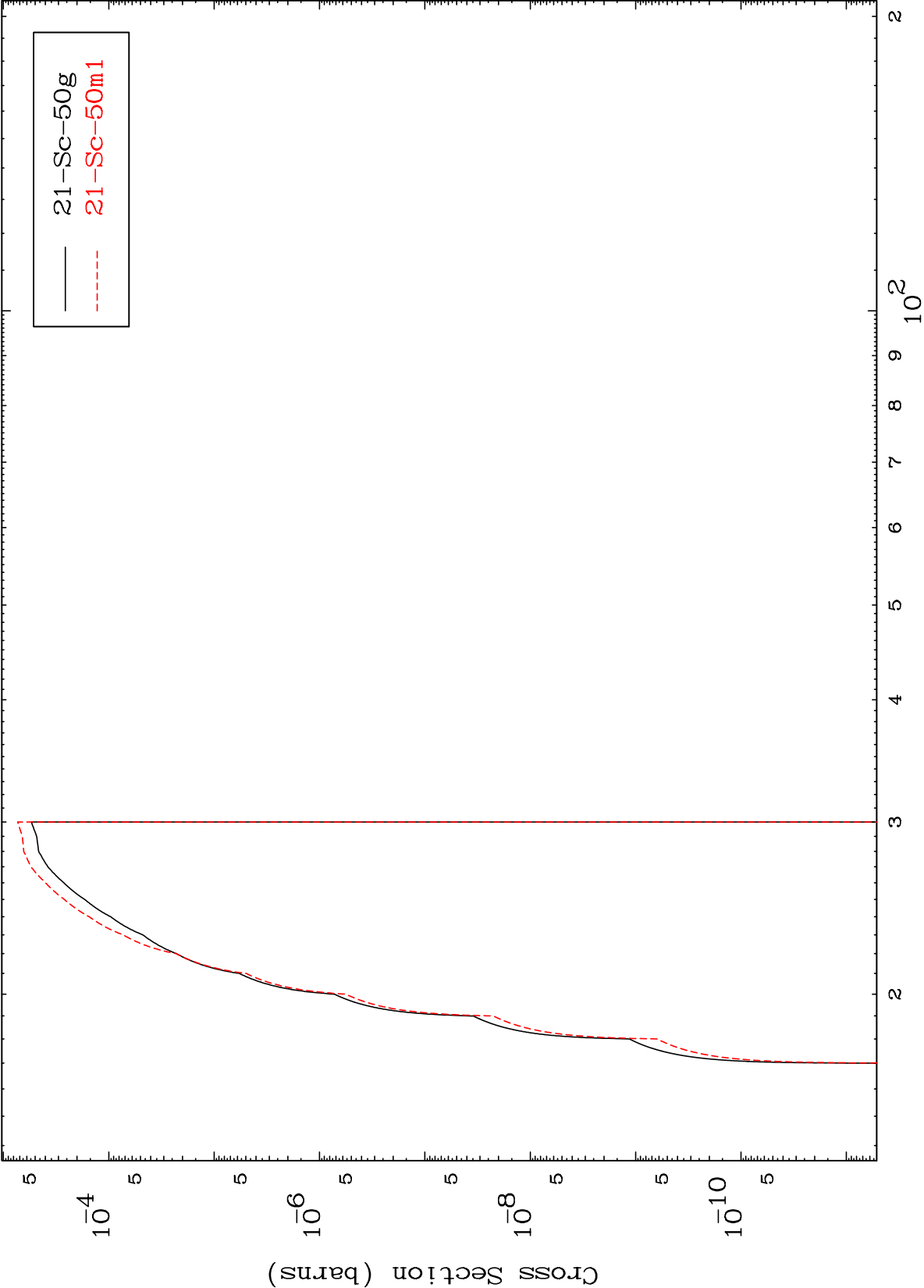




(n,2n) p
Radionuclide Production Cross Section



Radionuclide Production Cross Section

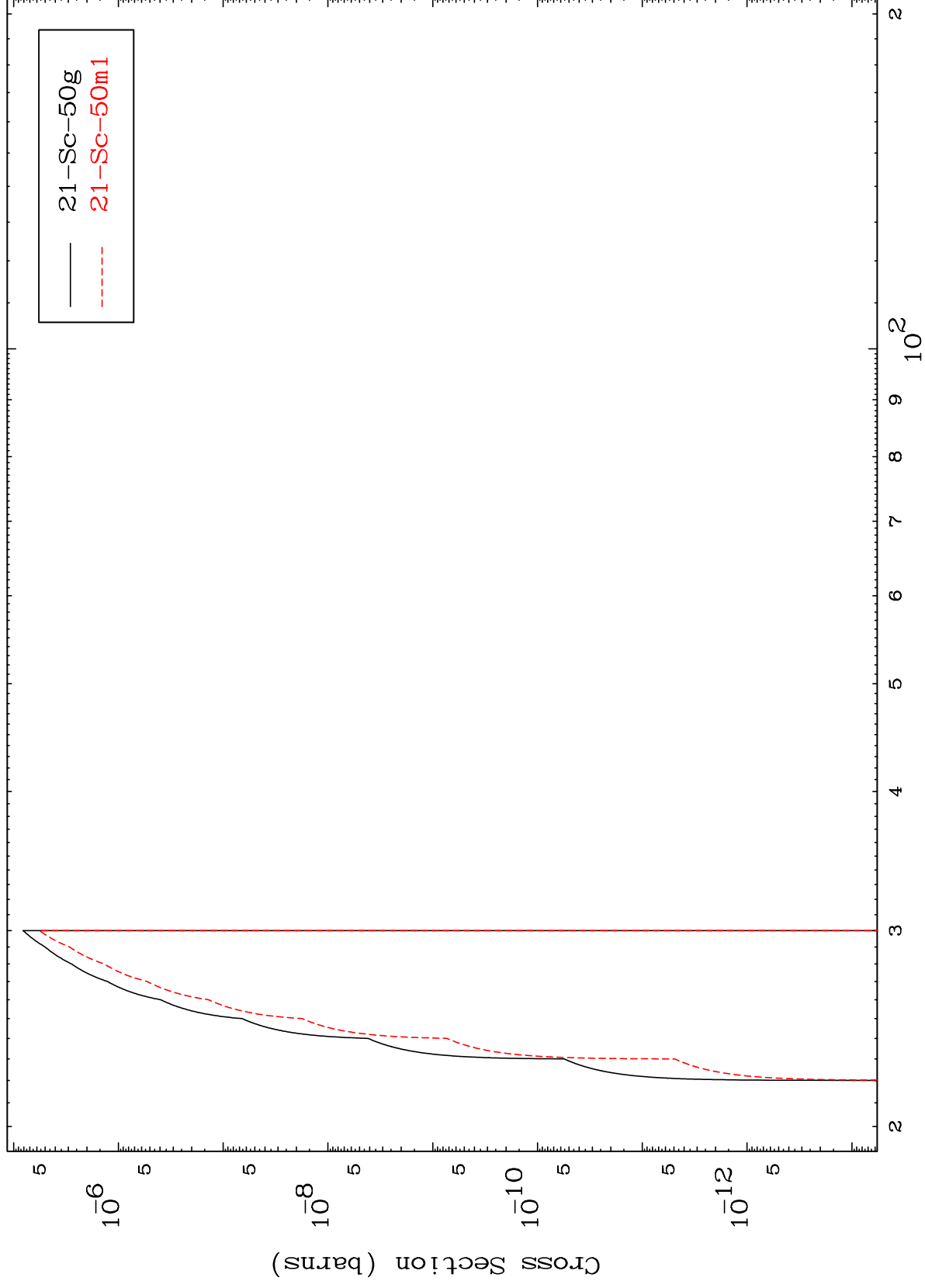


MAT 2331

(n,p) d

23-V -52

Radionuclide Production Cross Section



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Incident Energy (MeV)

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