

Program EVALPLOT
(Version 2021-1)

by

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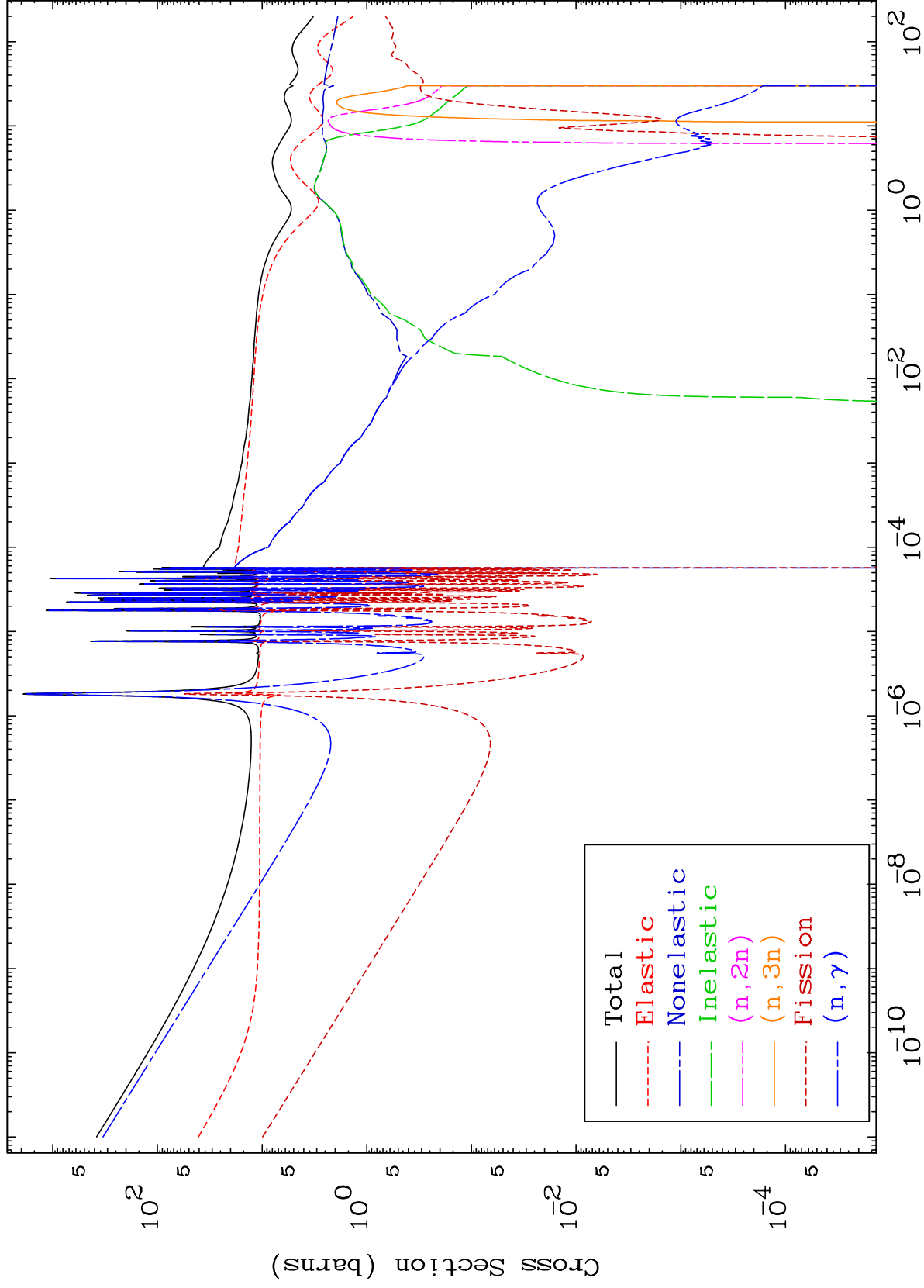
Press Mouse Button to Start

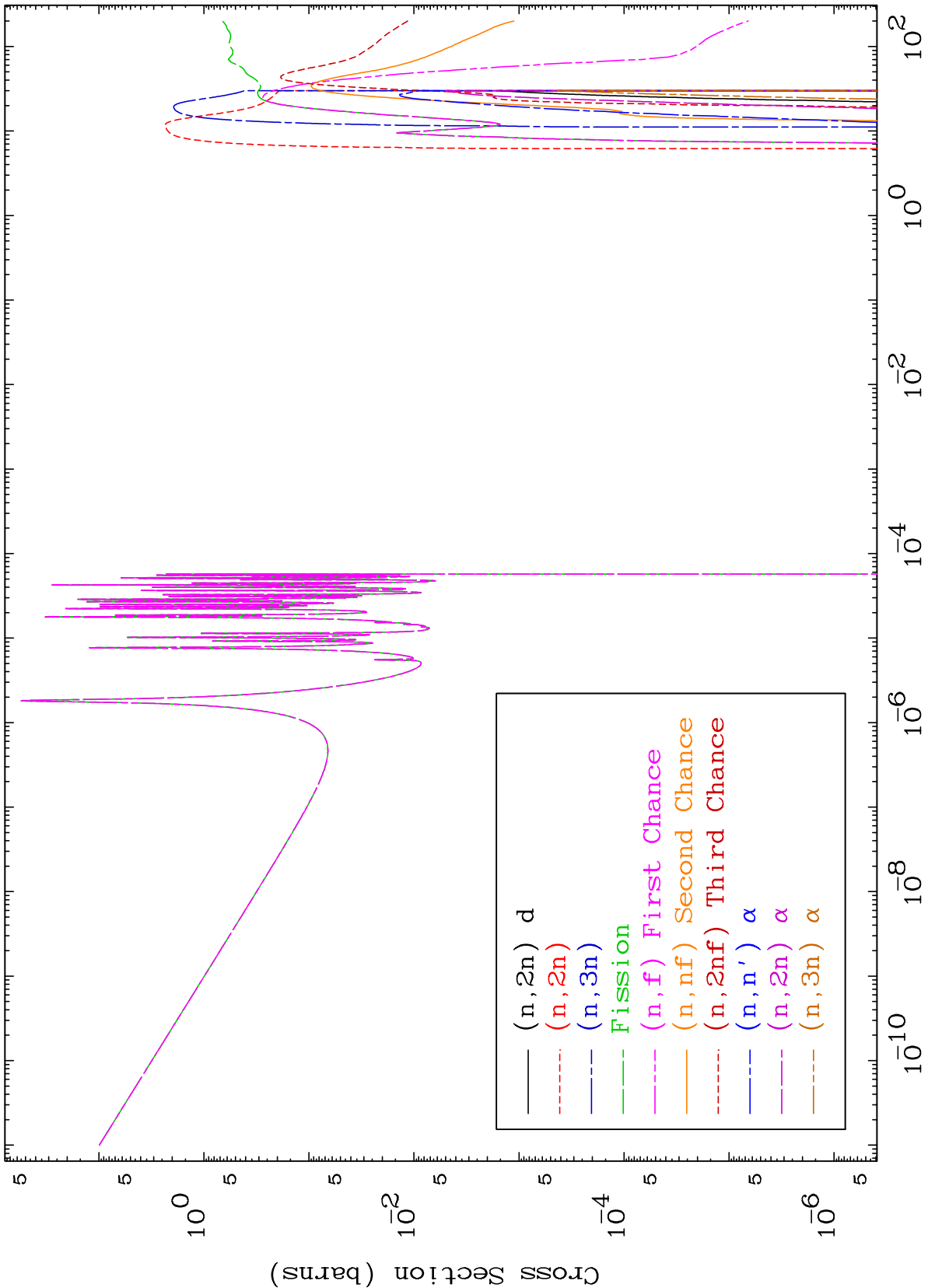
MAT 8943

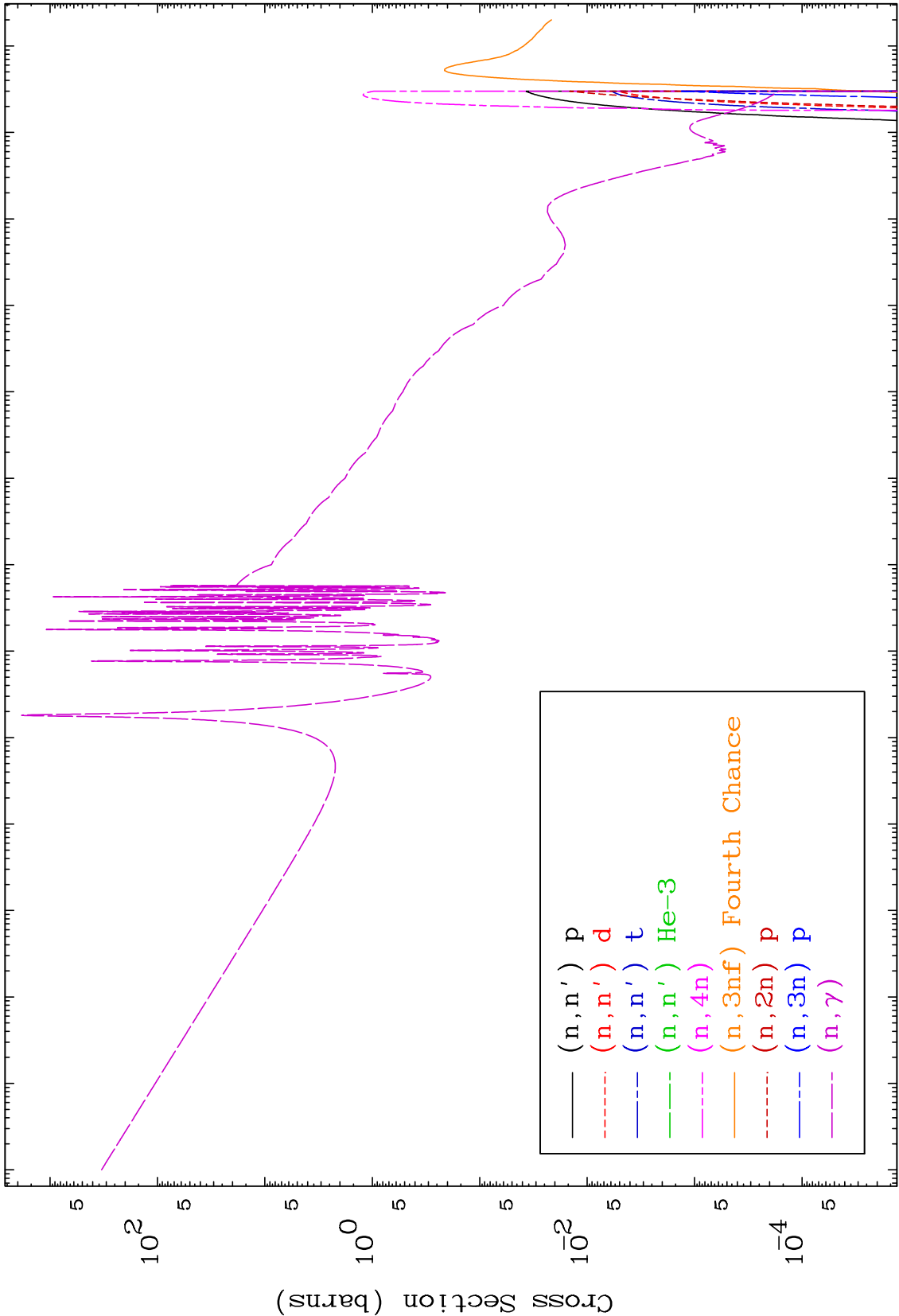
Neutron Major

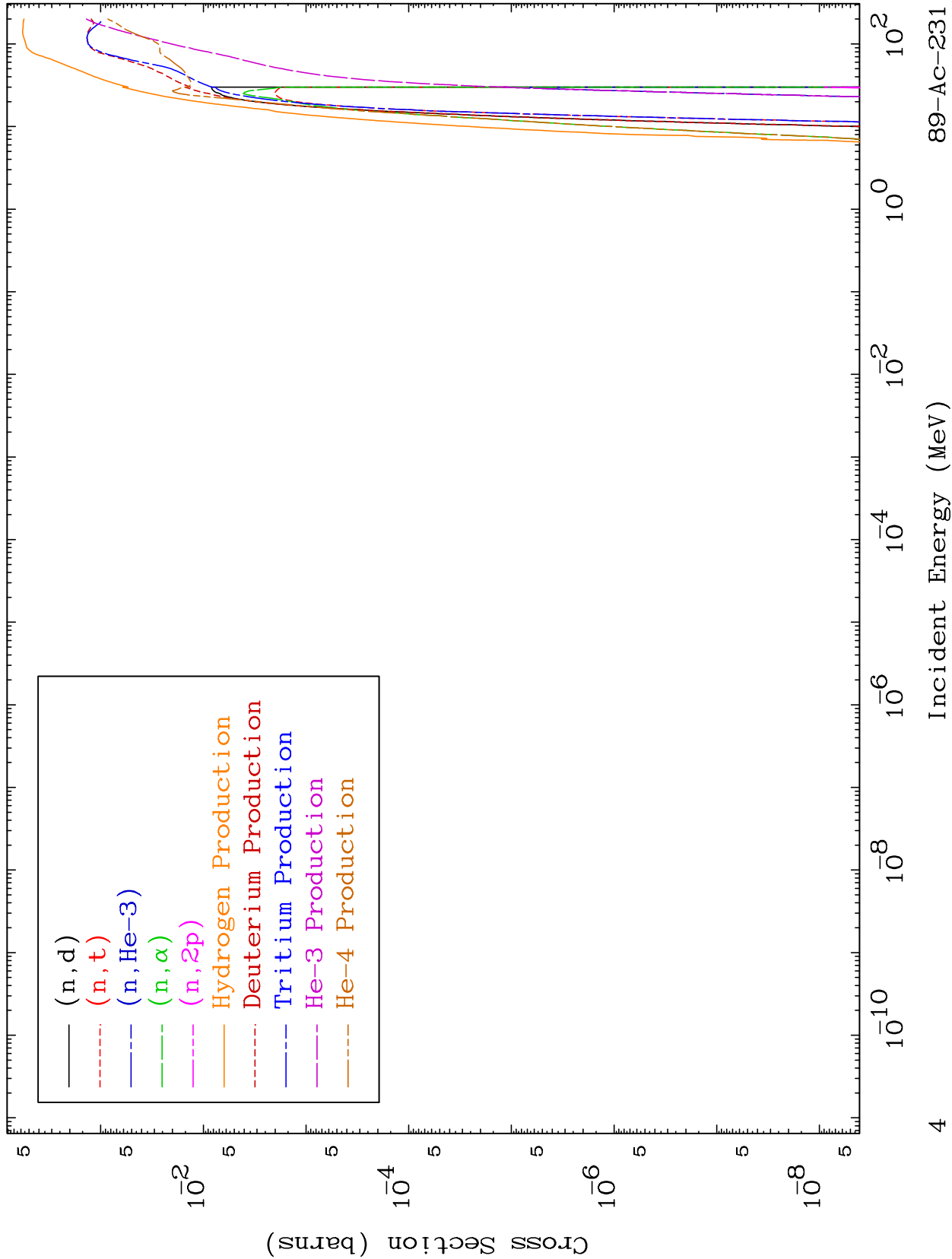
293 Kelvin Cross Sections

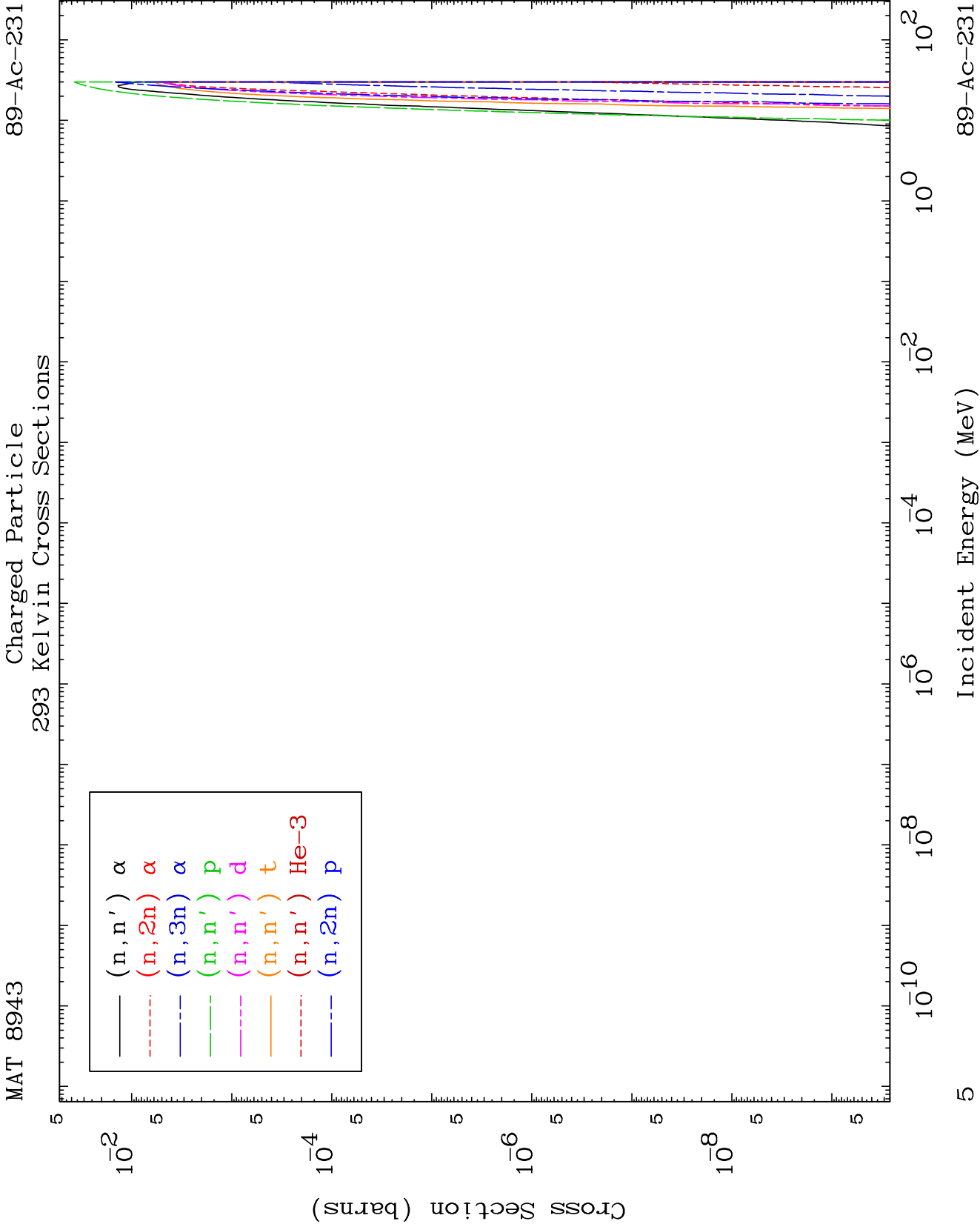
89-Ac-231

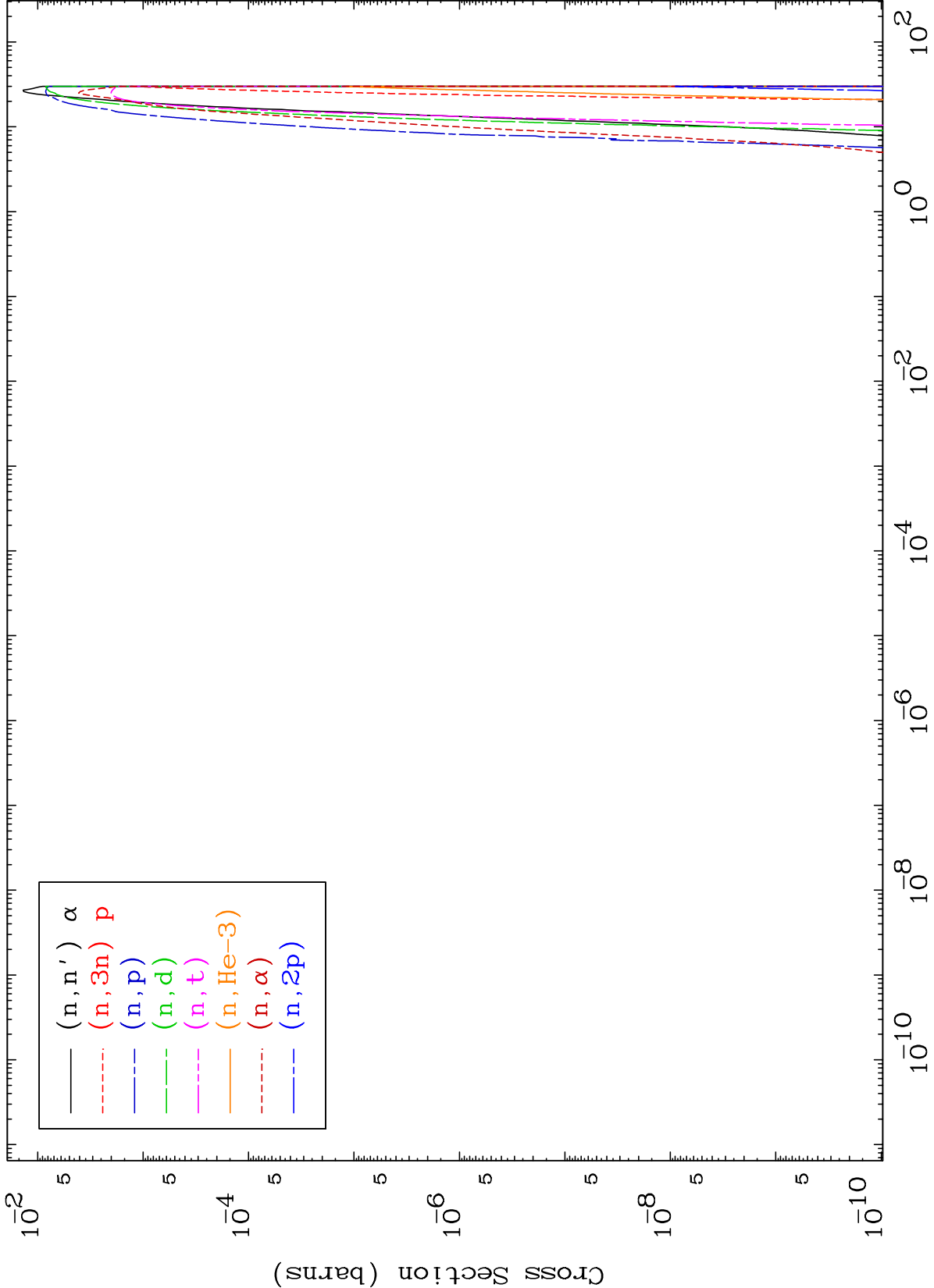


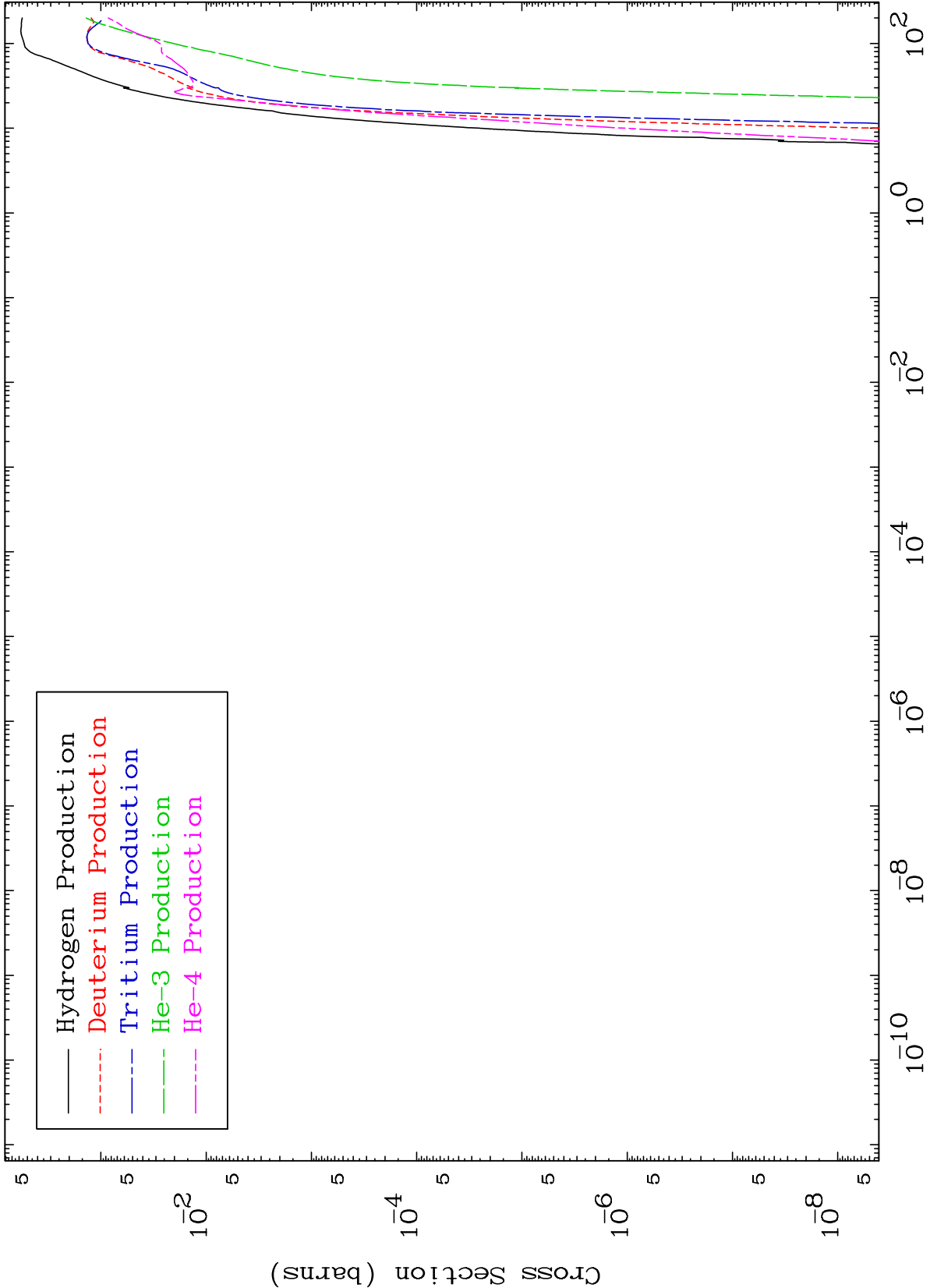










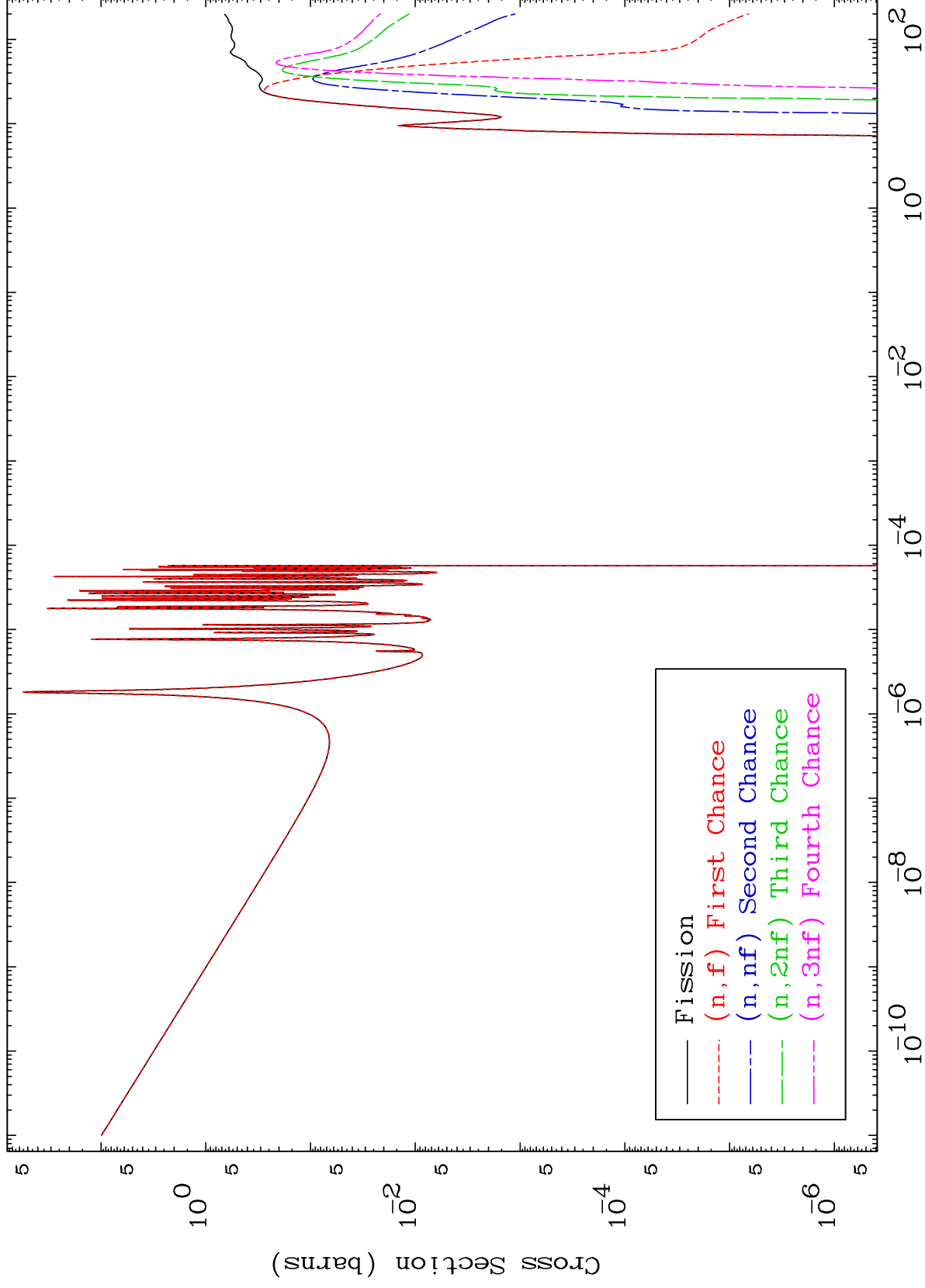


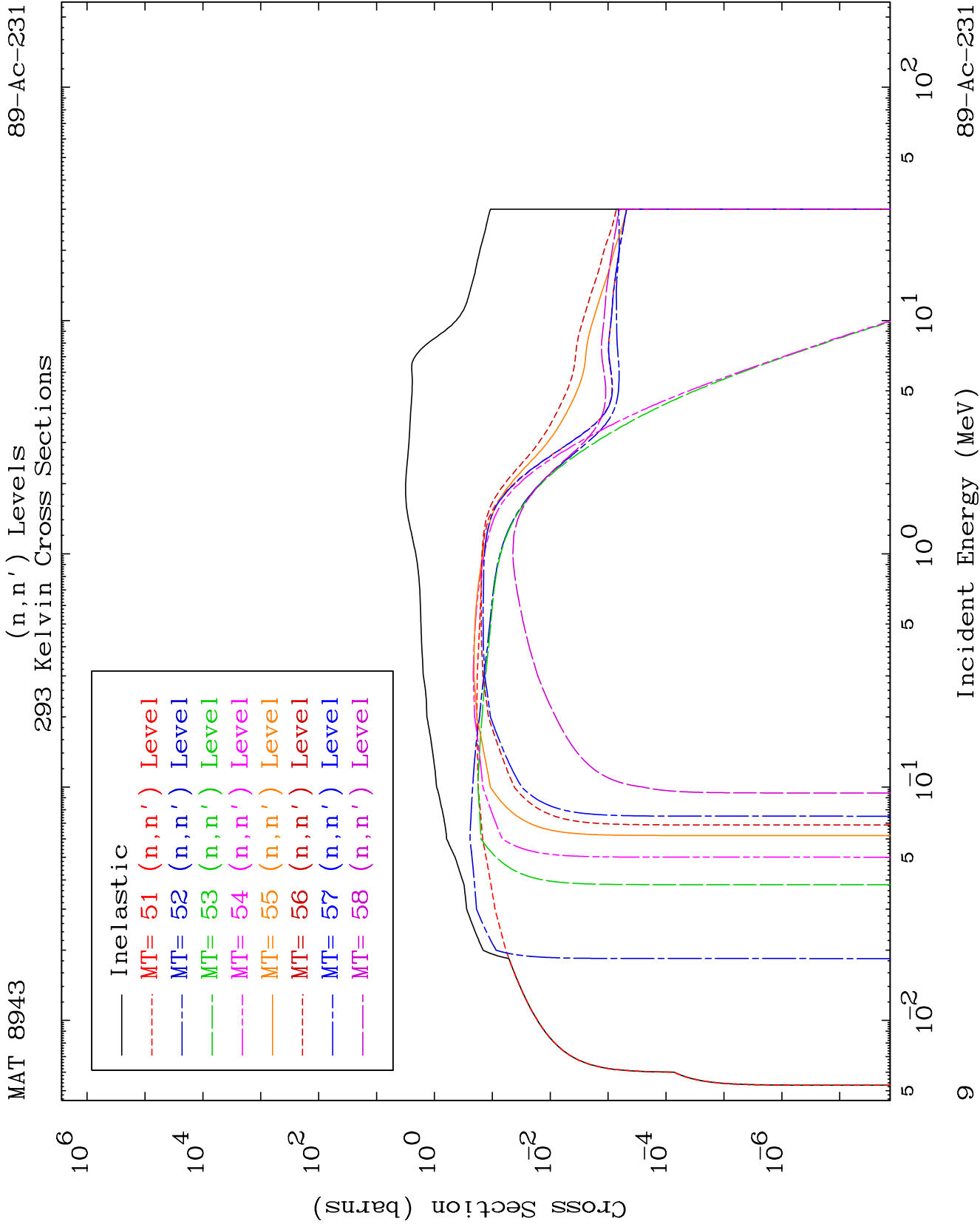
MAT 8943

Fission

293 Kelvin Cross Sections

89-Ac-231

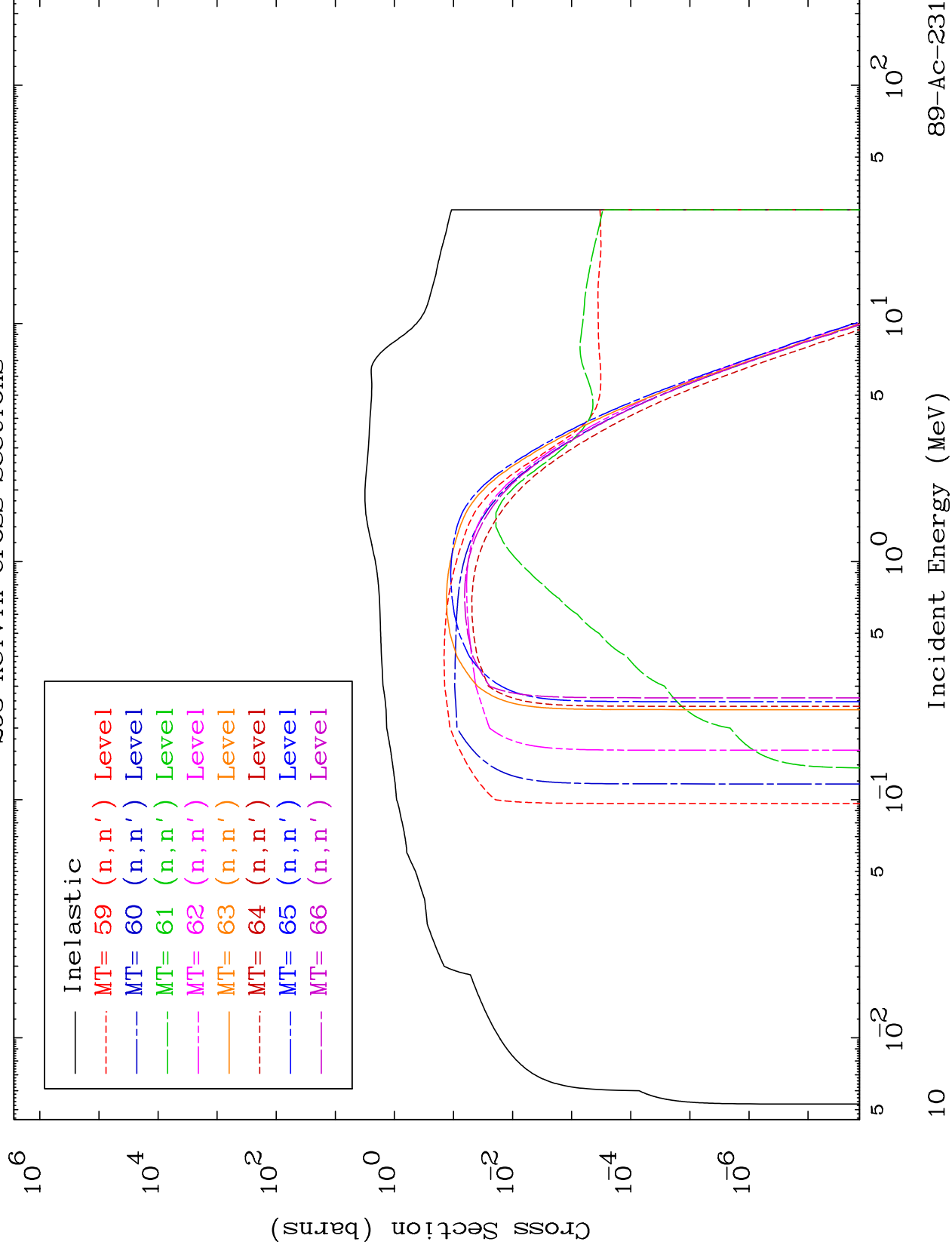


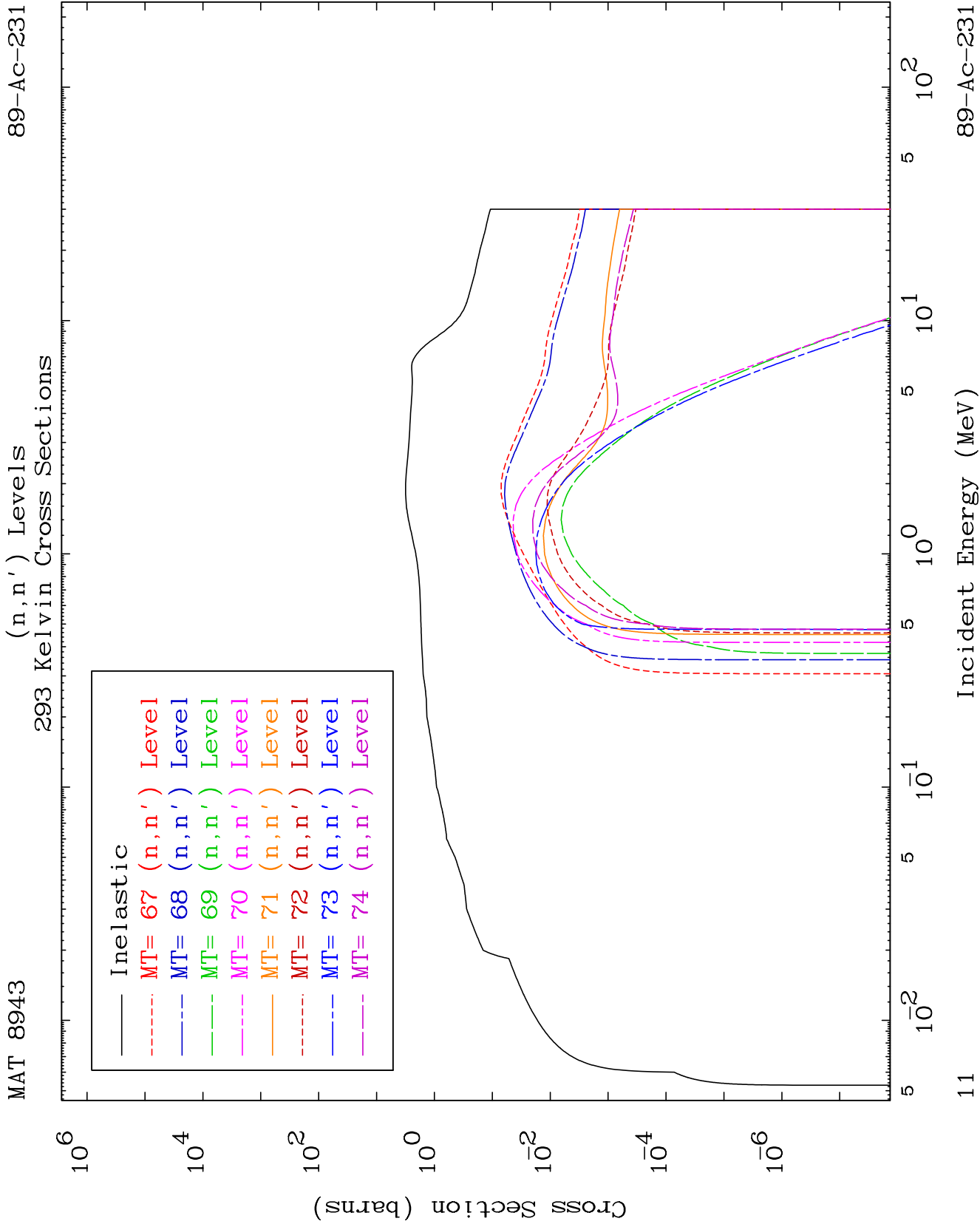


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89-Ac-231

(n, n') Levels
293 Kelvin Cross Sections

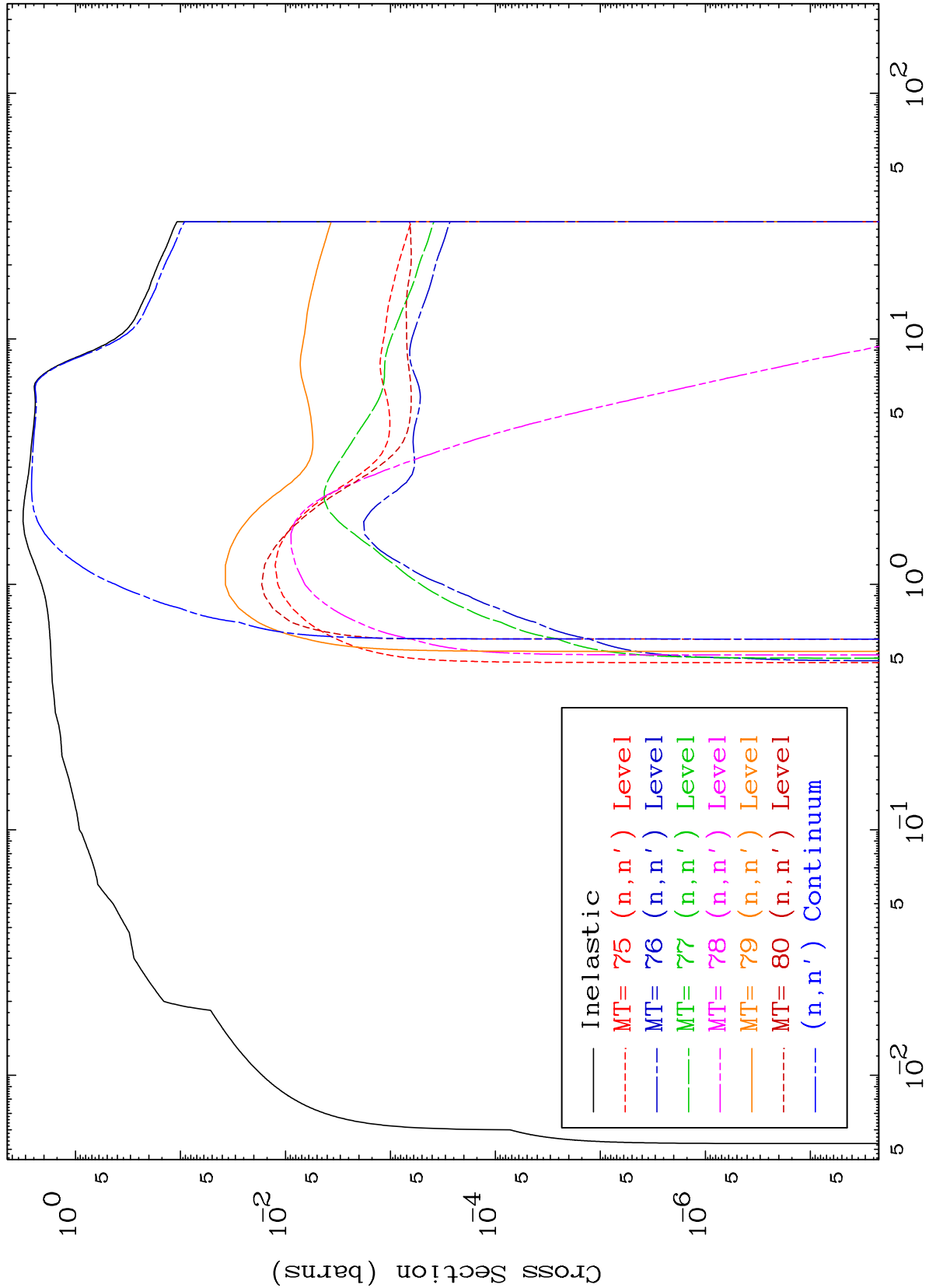




MAT 8943

(n,n') Levels
293 Kelvin Cross Sections

89-Ac-231



89-Ac-231

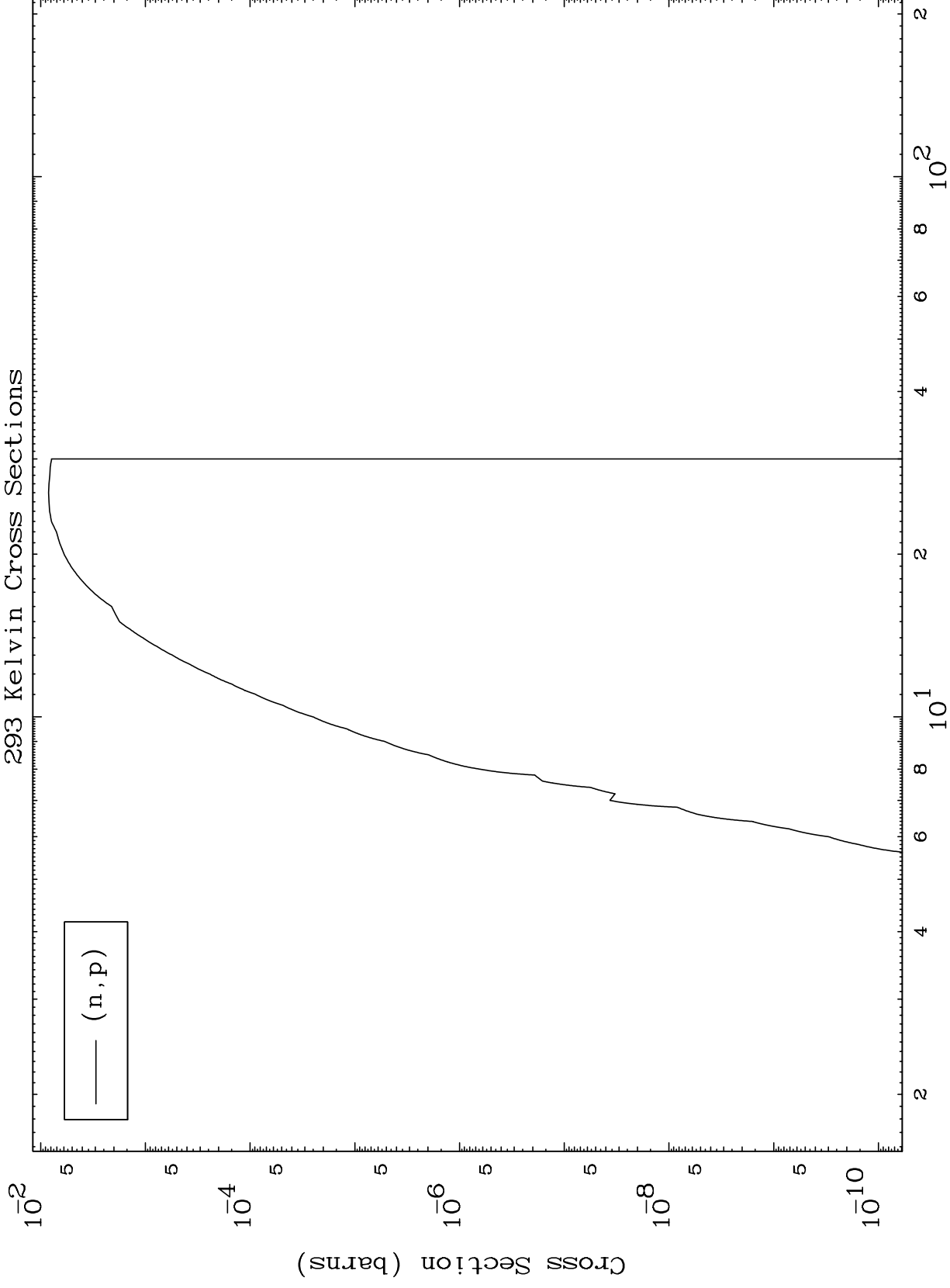
Incident Energy (MeV)

12

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(n,p) Levels
293 Kelvin Cross Sections

89-Ac-231



13

Incident Energy (MeV)

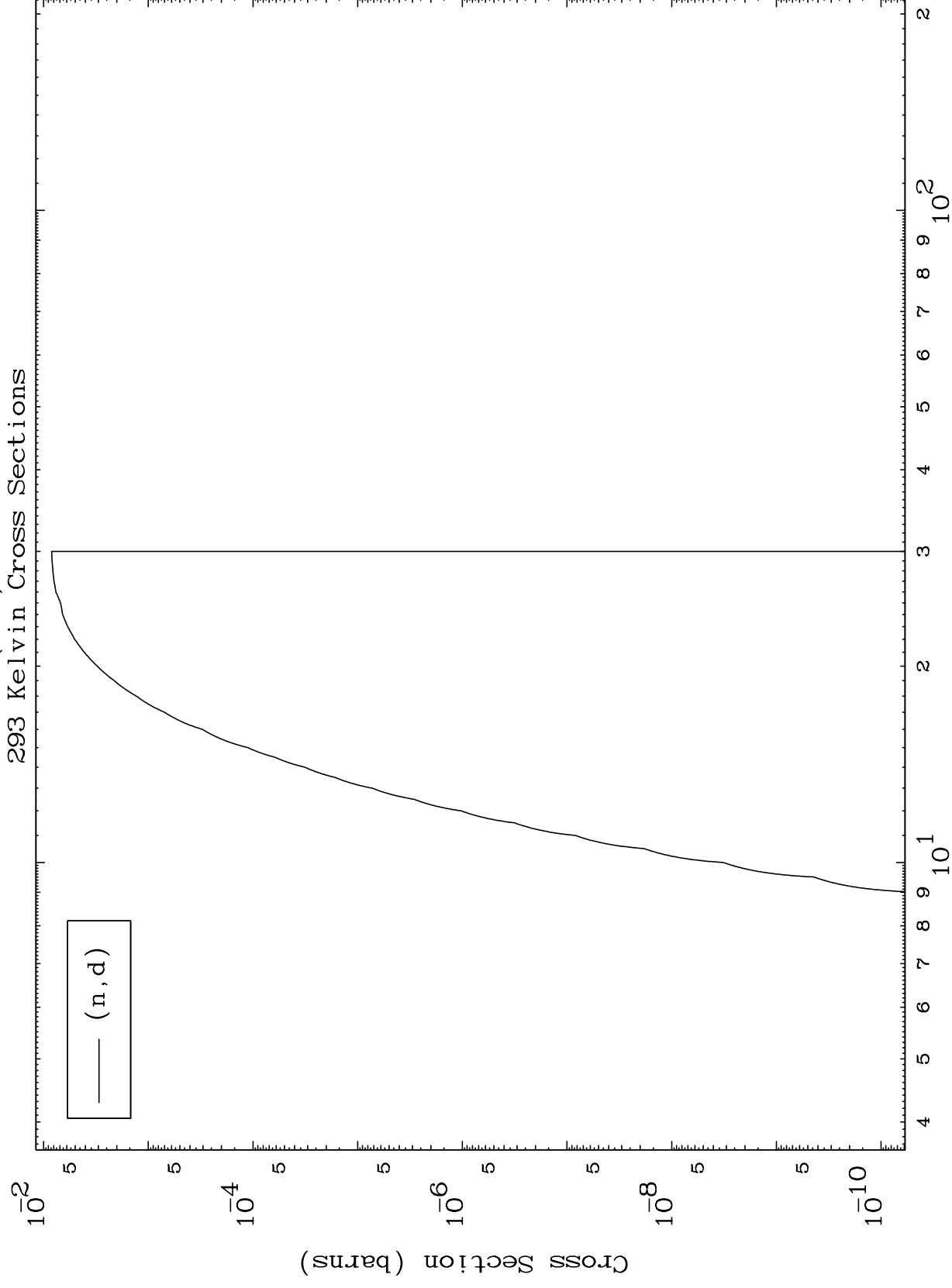
89-Ac-231

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(n,d) Levels

89-AC-231

293 Kelvin Cross Sections



14

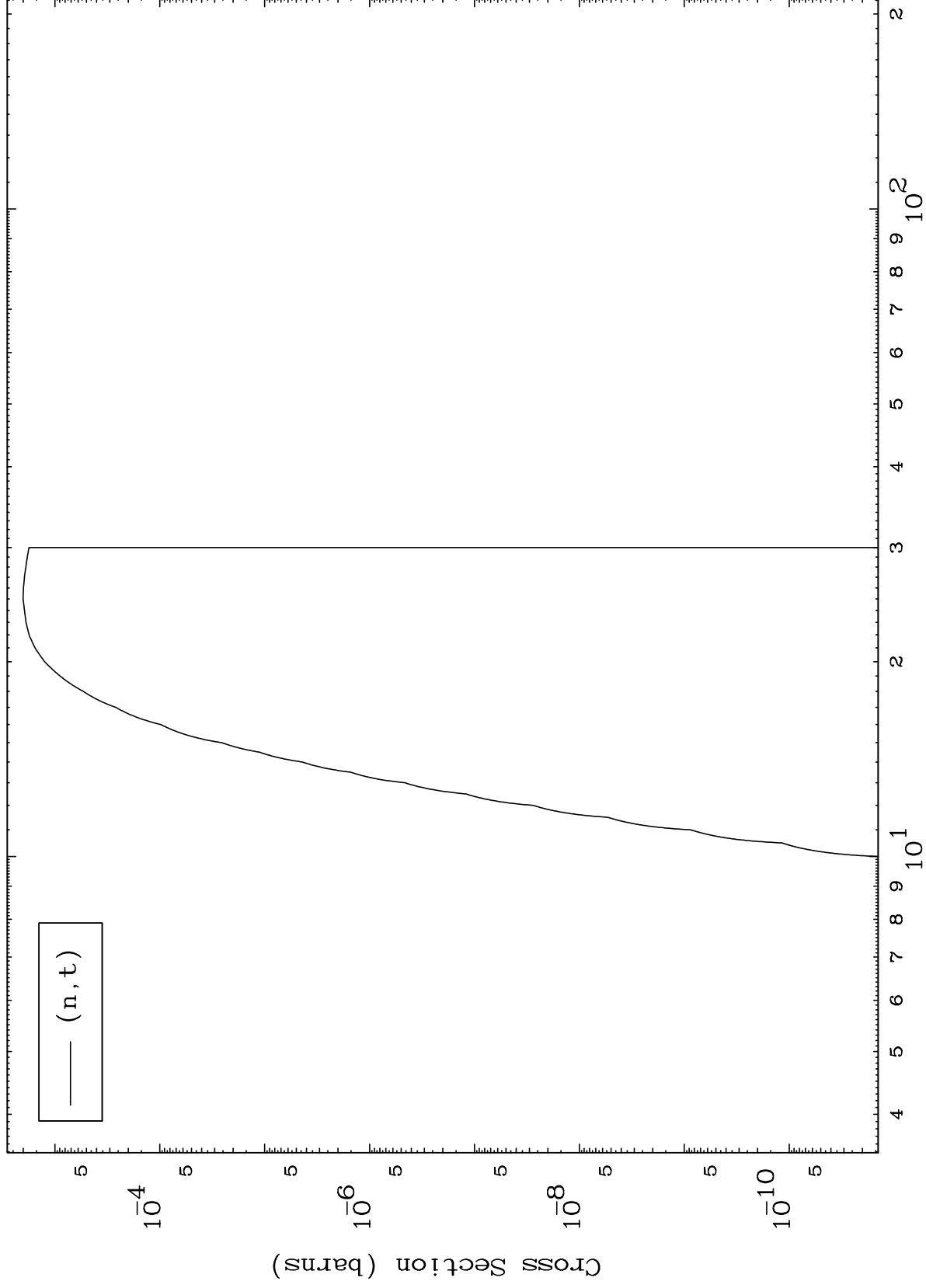
Incident Energy (MeV)

89-Ac-231

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89-Ac-231

(n,t) Levels
293 Kelvin Cross Sections



89-Ac-231

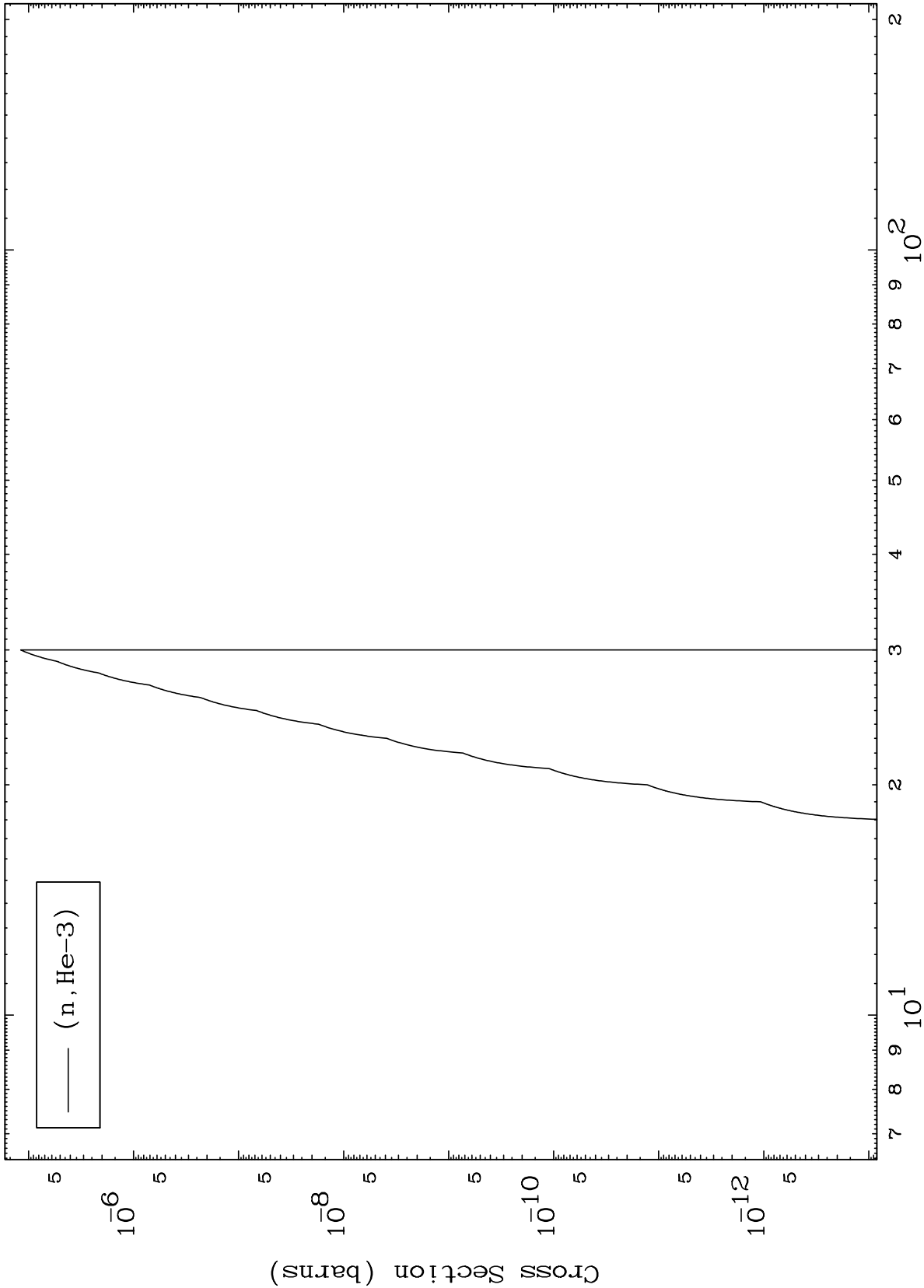
Incident Energy (MeV)

15

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(n,He3) Levels
293 Kelvin Cross Sections

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16

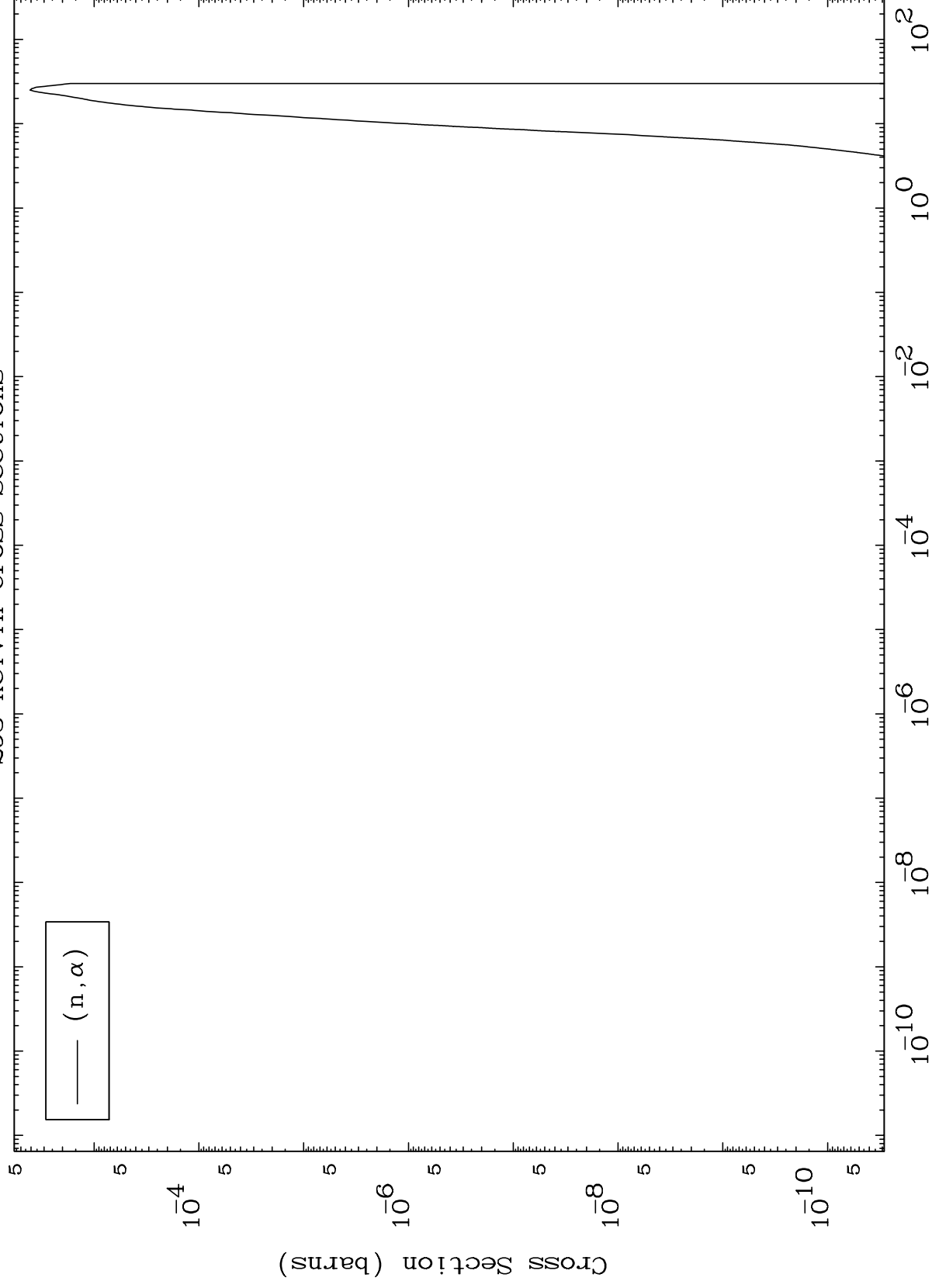
Incident Energy (MeV)

89-Ac-231

MAT 8943

(n, α) Levels
293 Kelvin Cross Sections

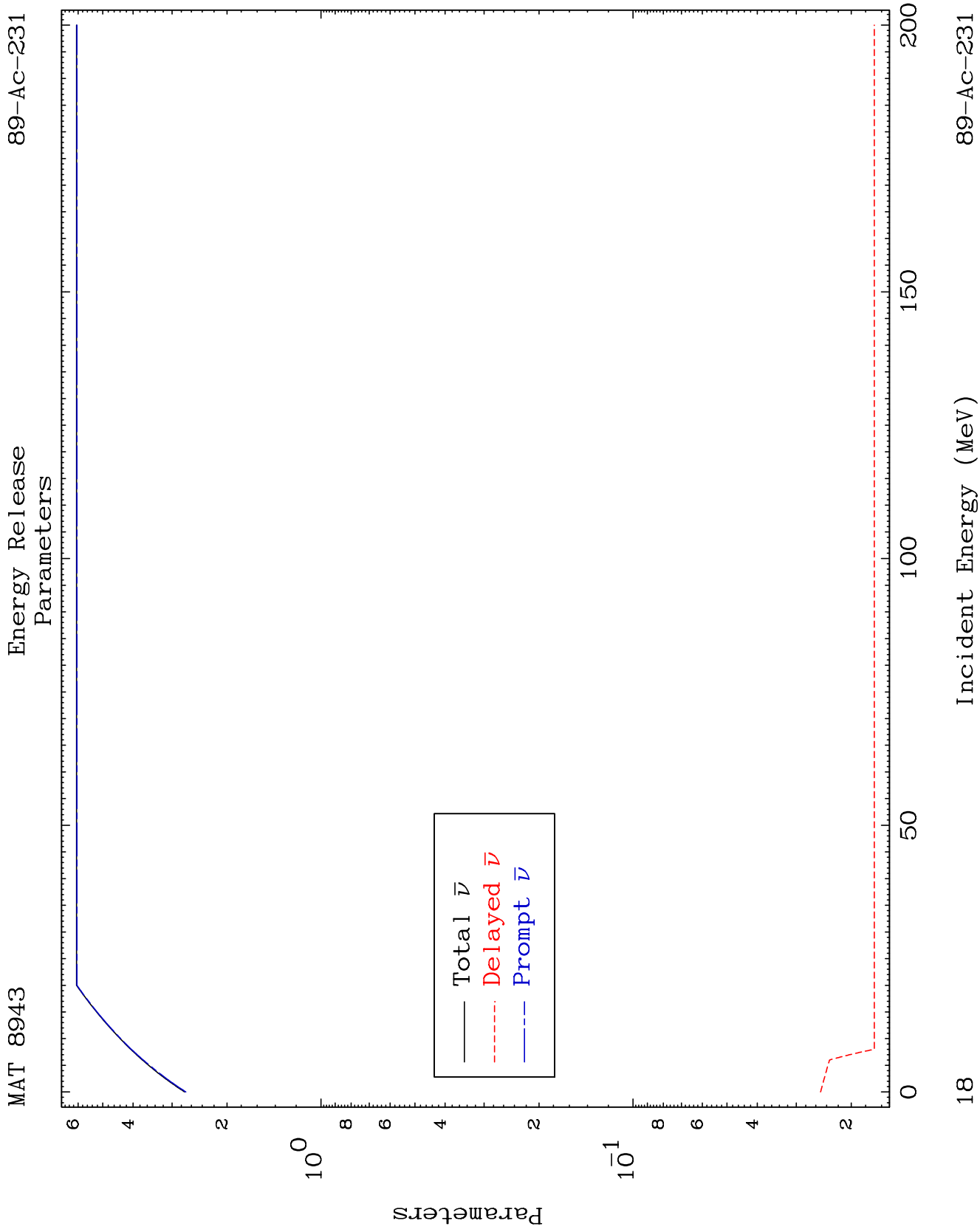
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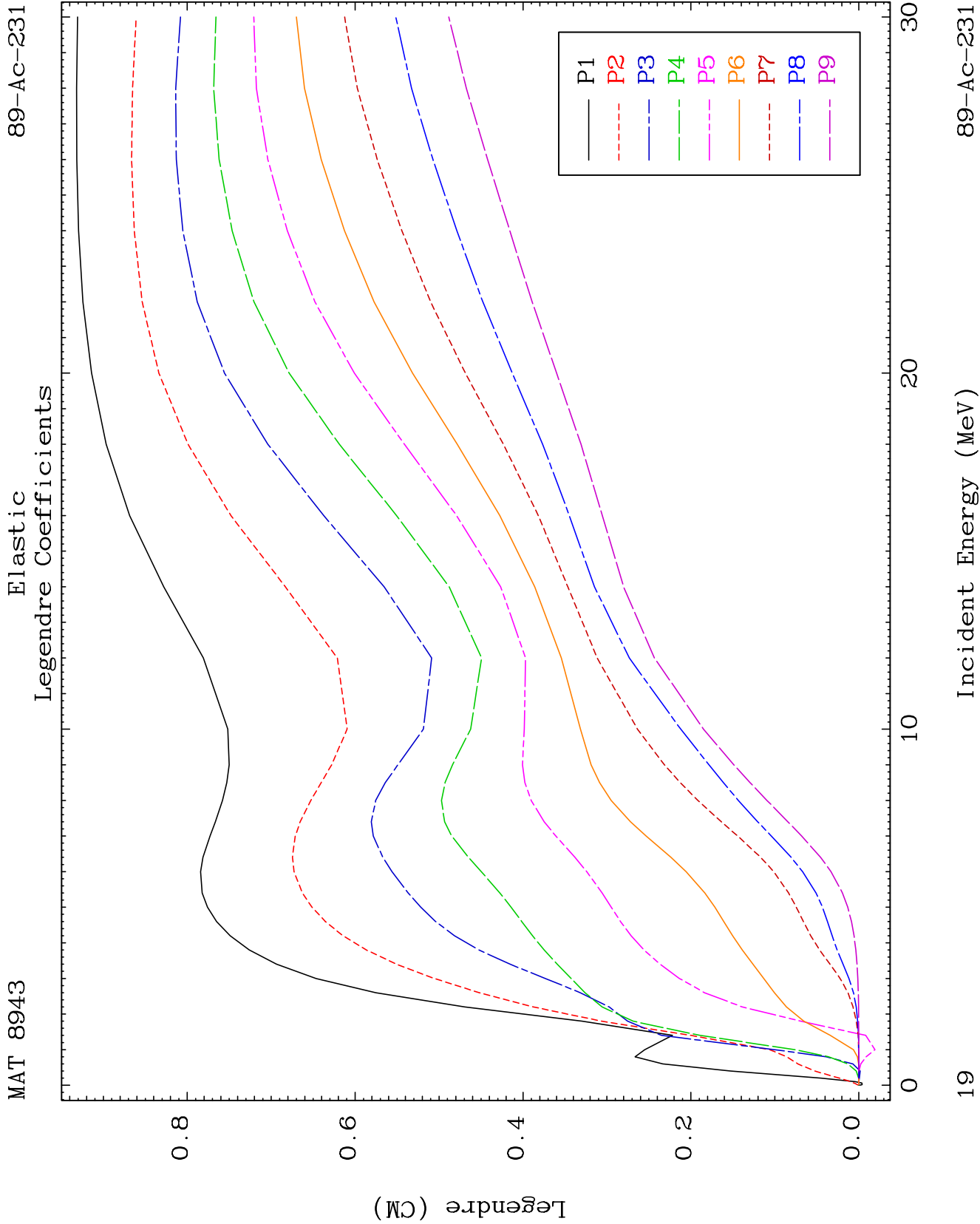


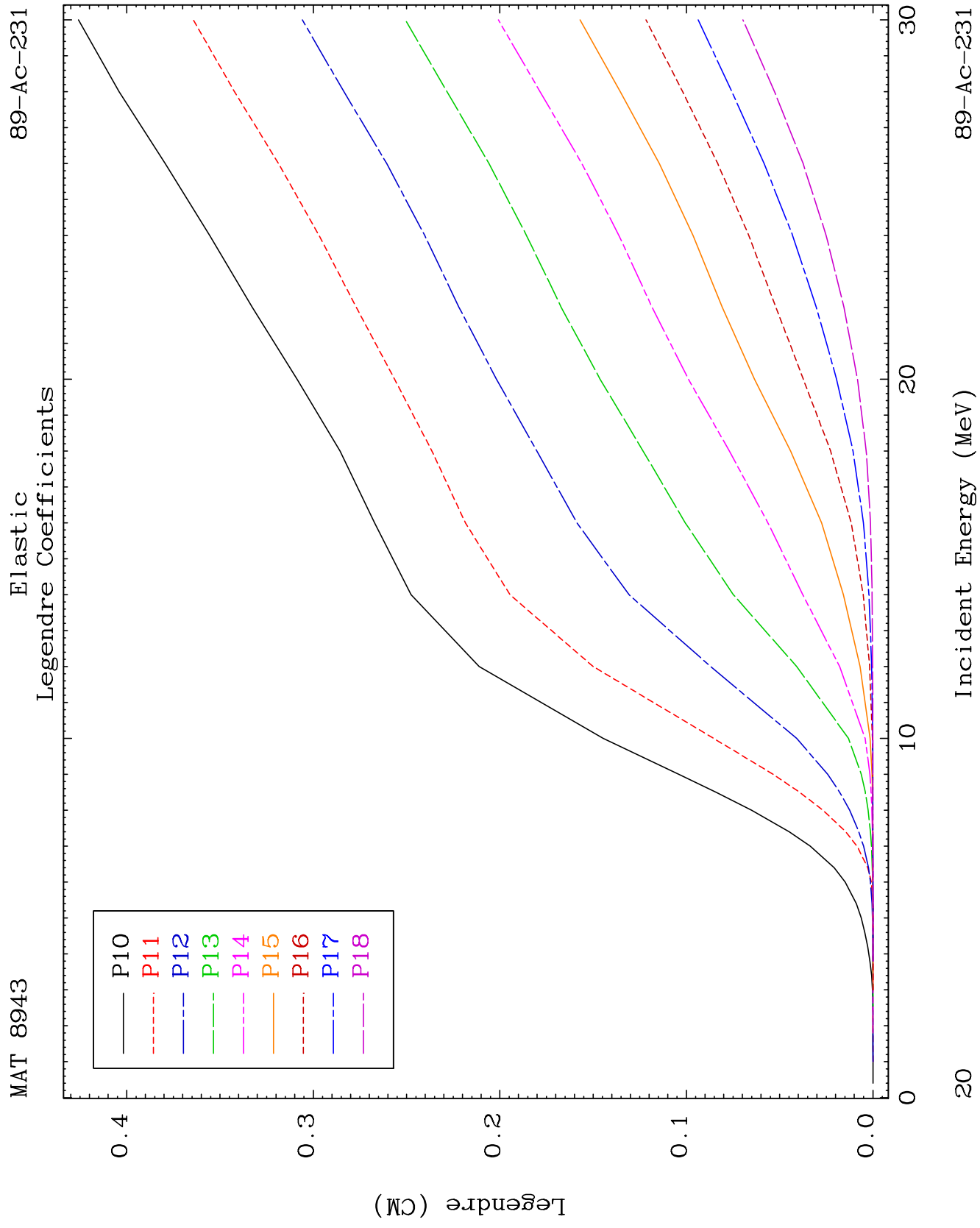
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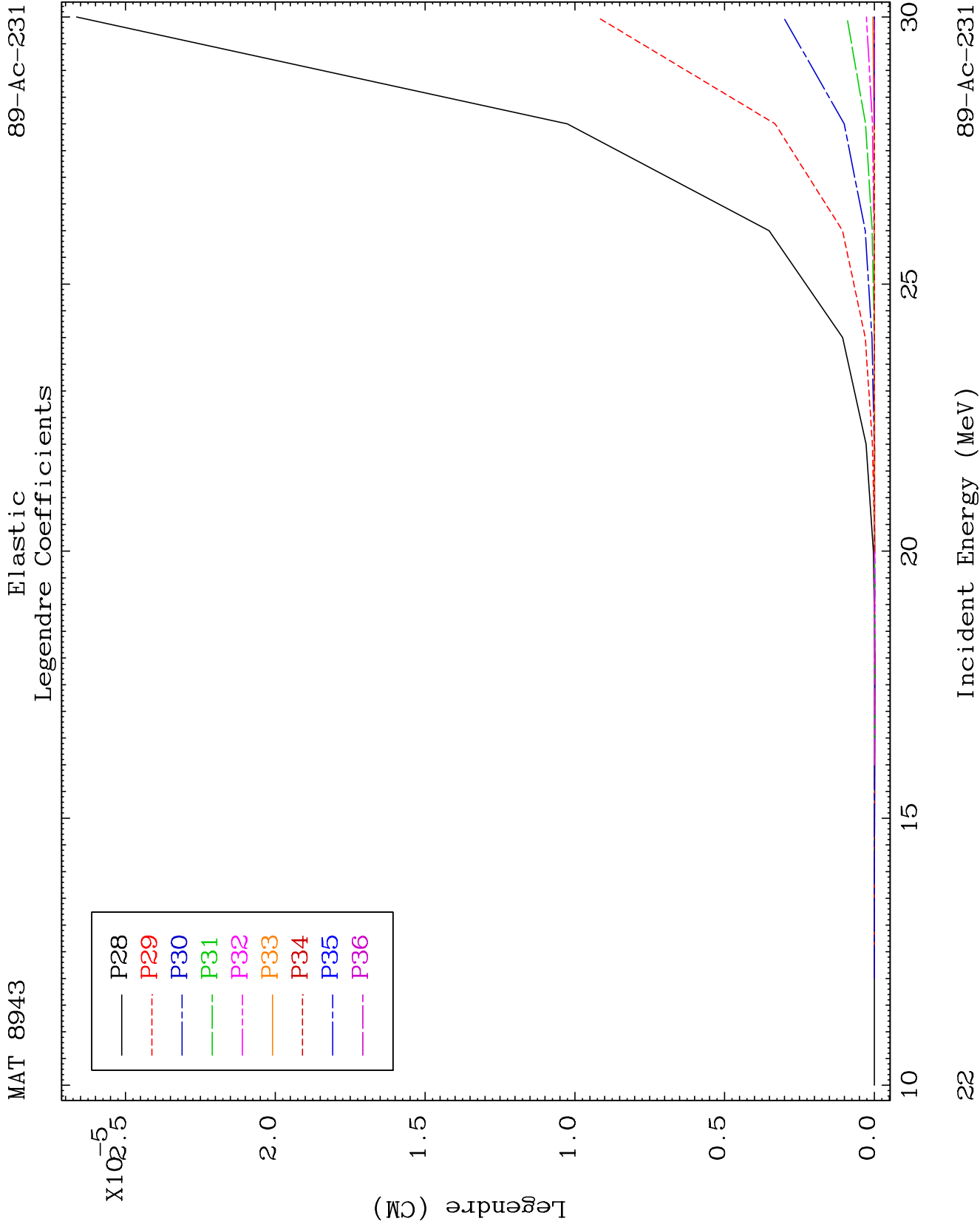
Incident Energy (MeV)

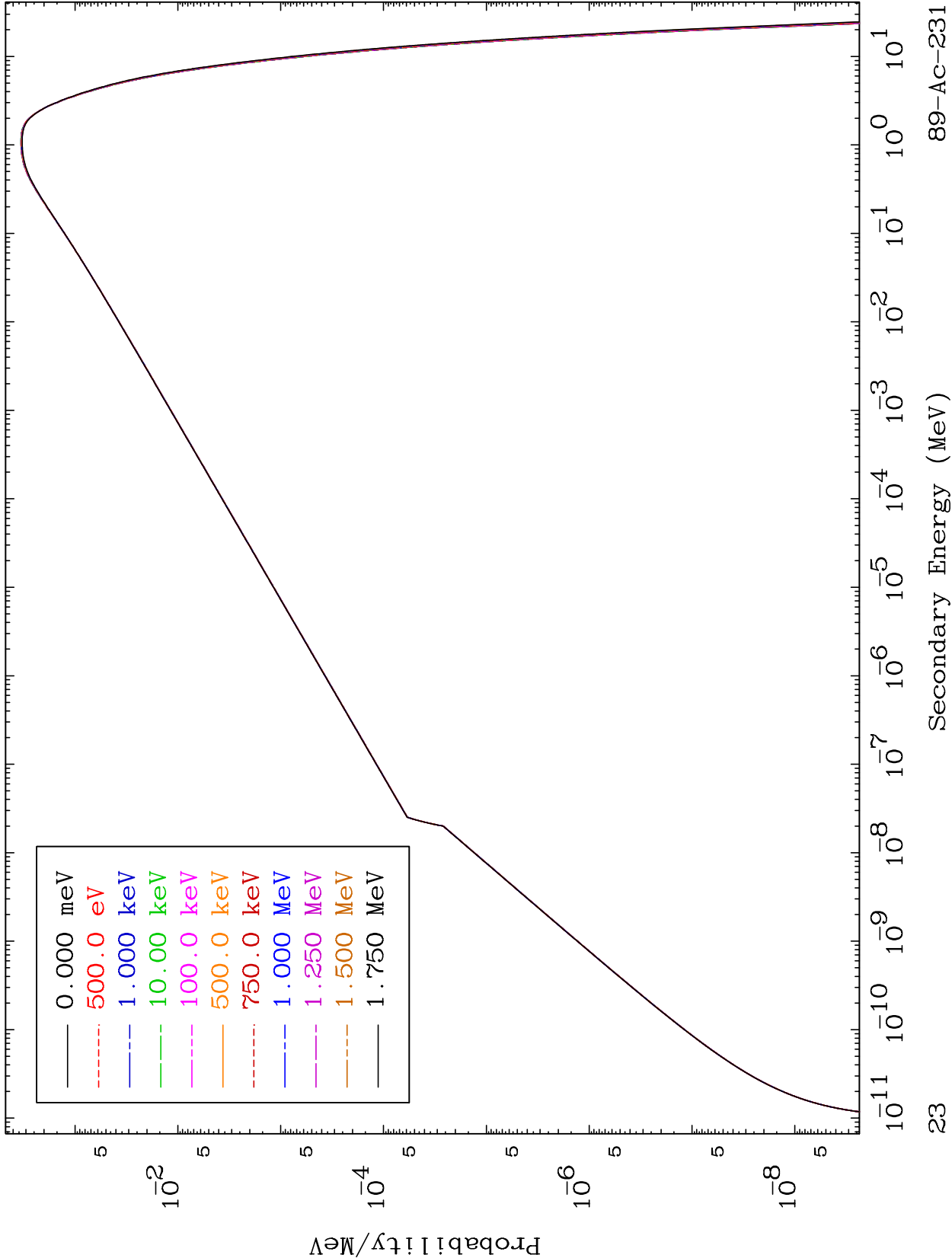
89-Ac-231



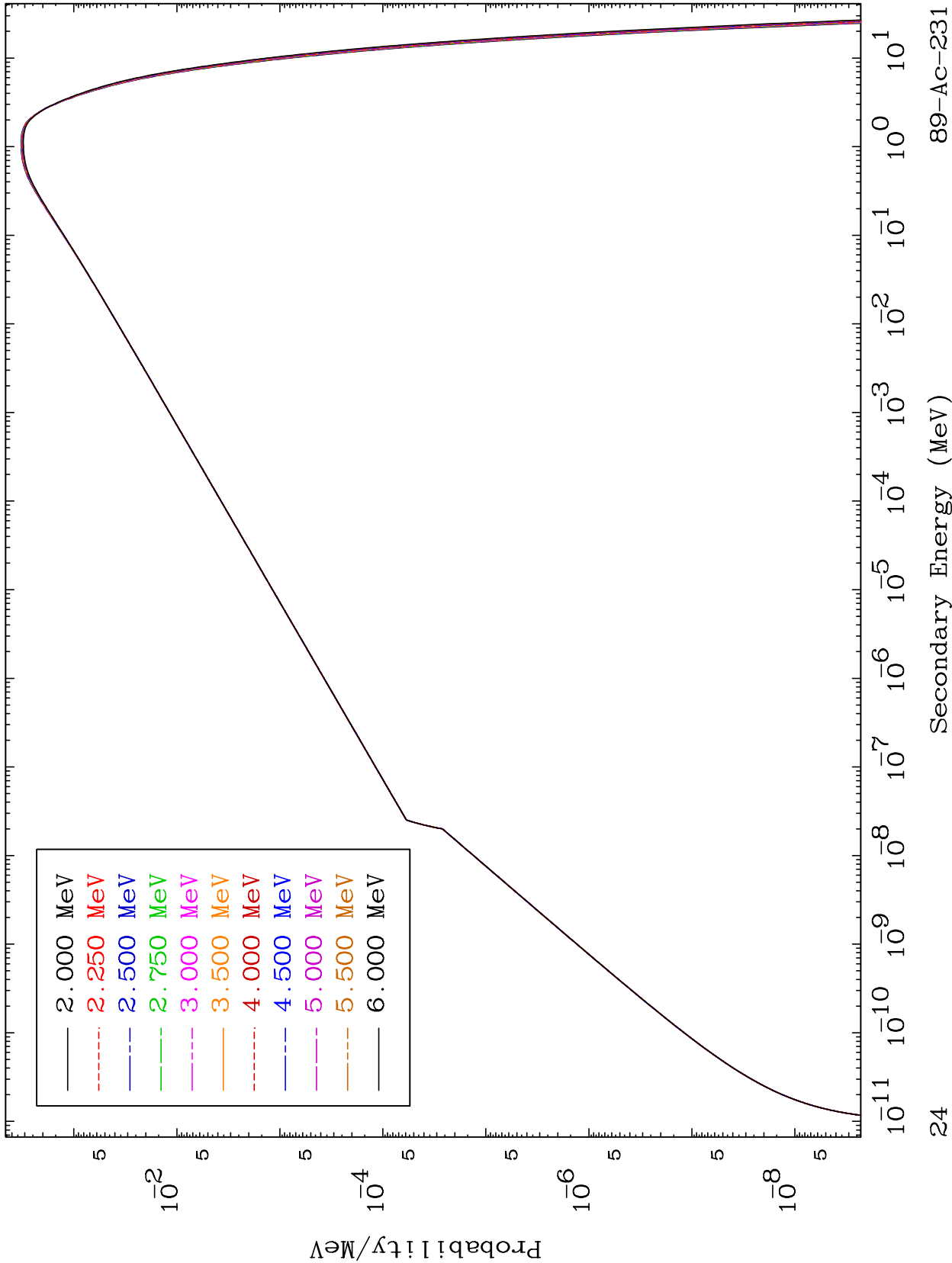


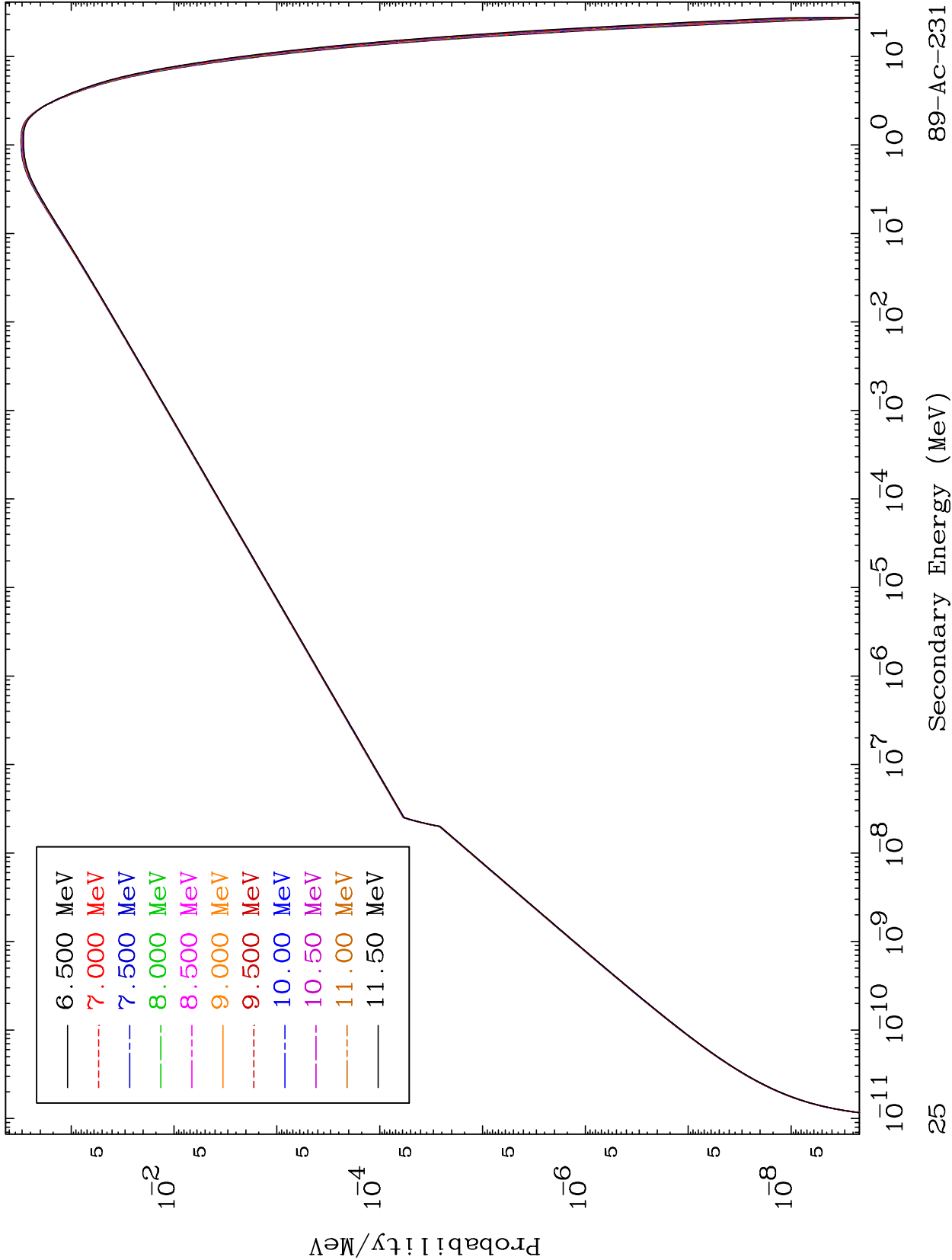


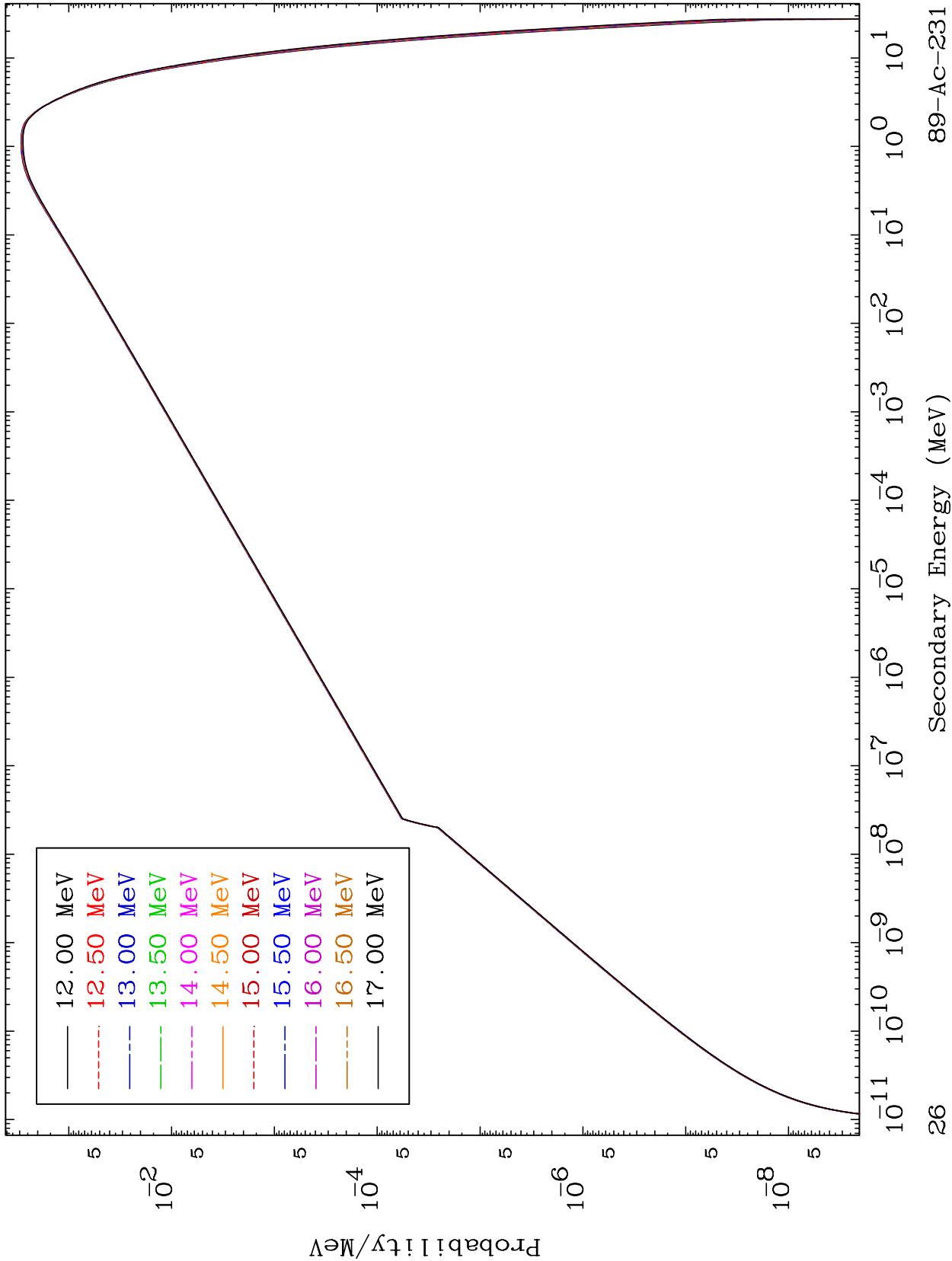




Fission
Energy Distributions







Fission
Energy Distributions

