

Program EVALPLOT  
(Version 2021-1)

by

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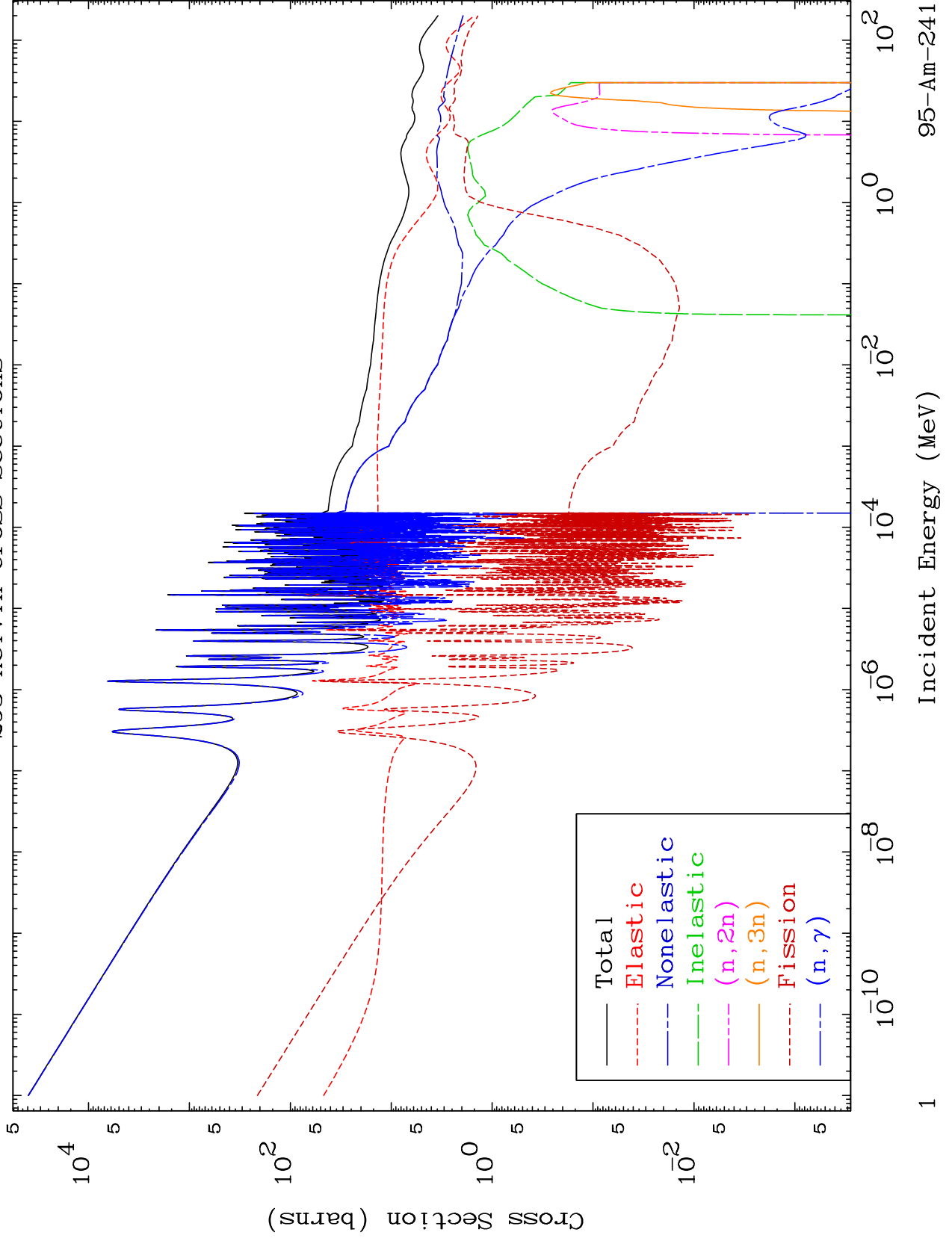
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 9543

Neutron Major  
293 Kelvin Cross Sections

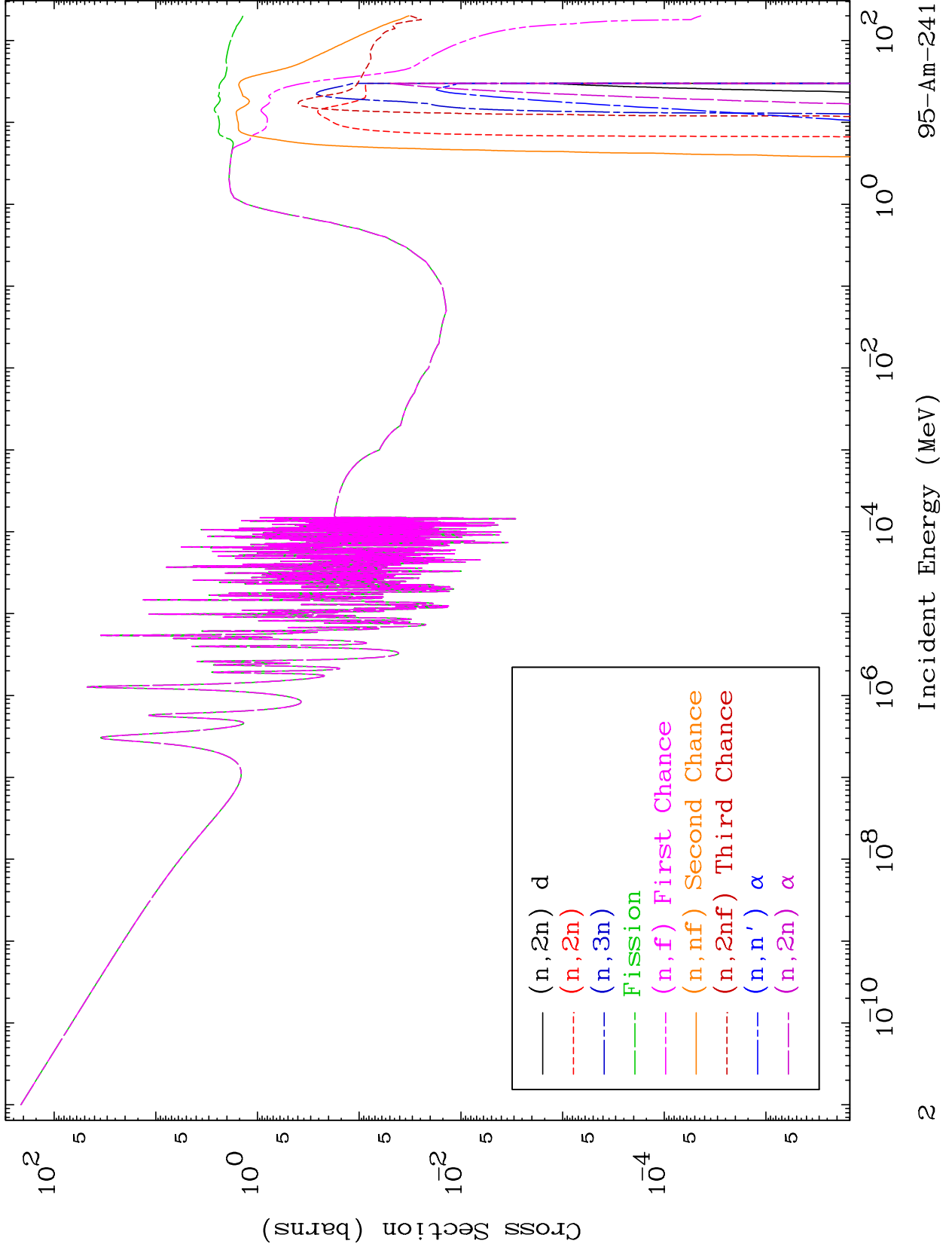
95-Am-241



MAT 9543

Neutron Absorption  
293 Kelvin Cross Sections

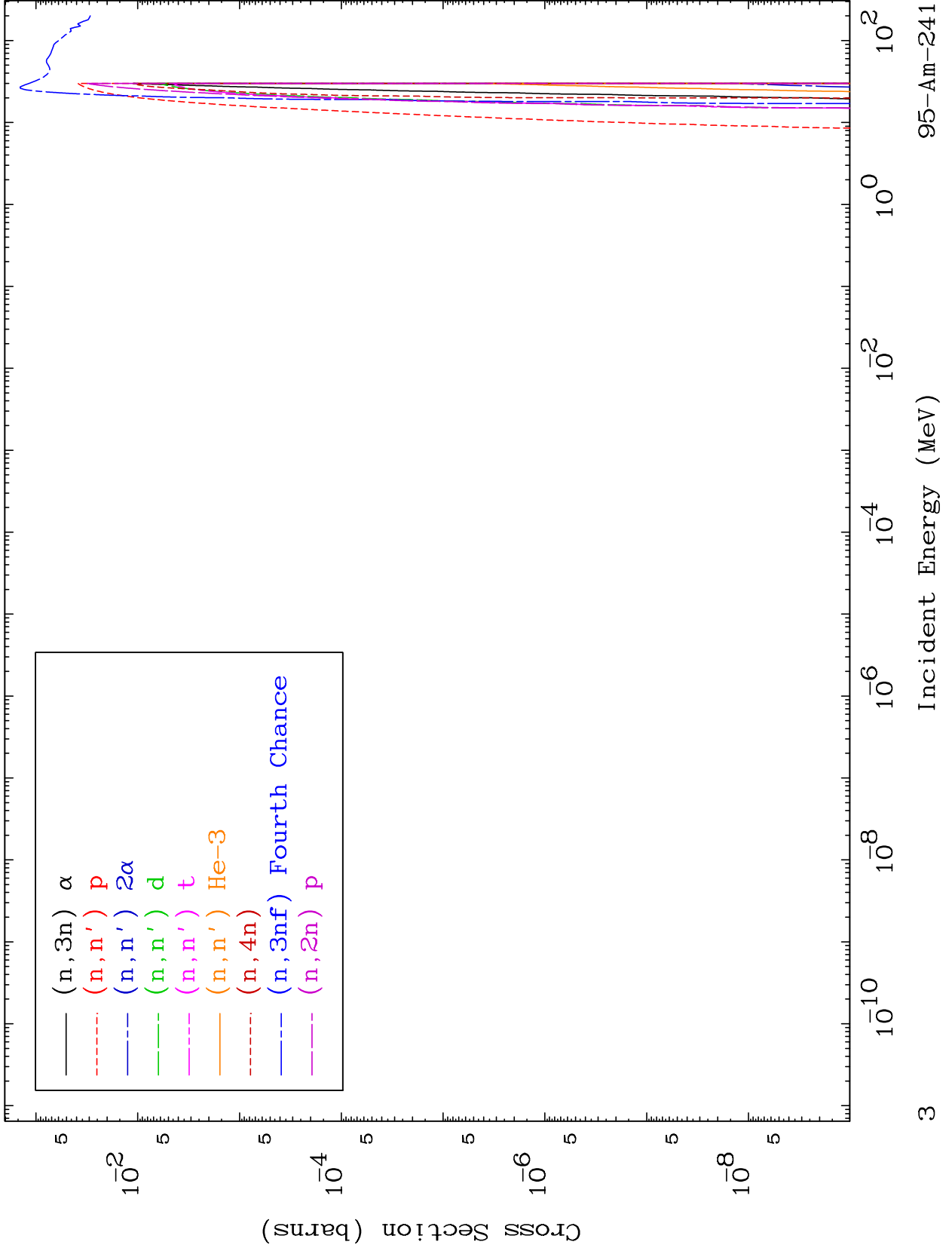
95-Am-241



MAT 9543

Neutron Absorption  
293 Kelvin Cross Sections

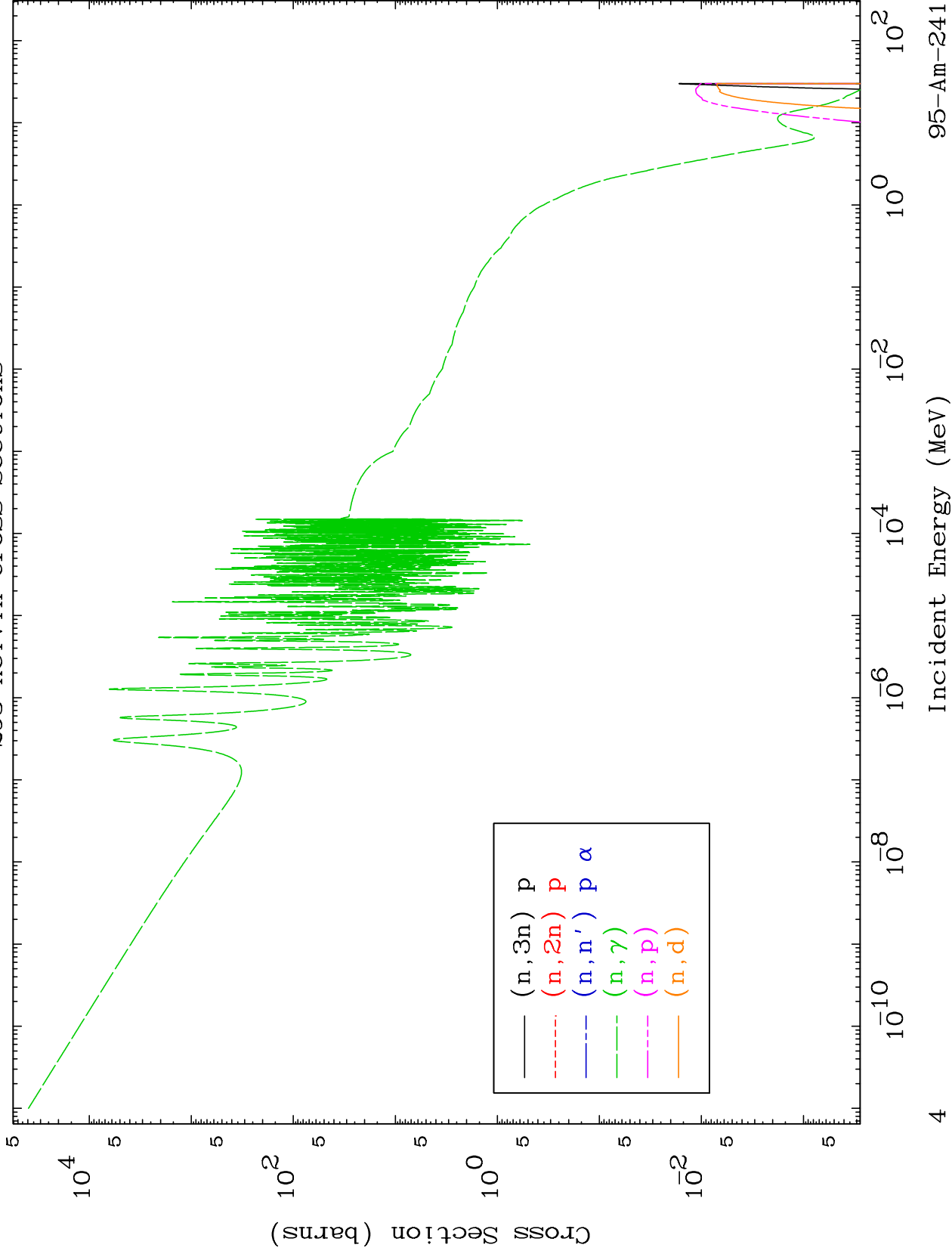
95-Am-241



MAT 9543

Neutron Absorption  
293 Kelvin Cross Sections

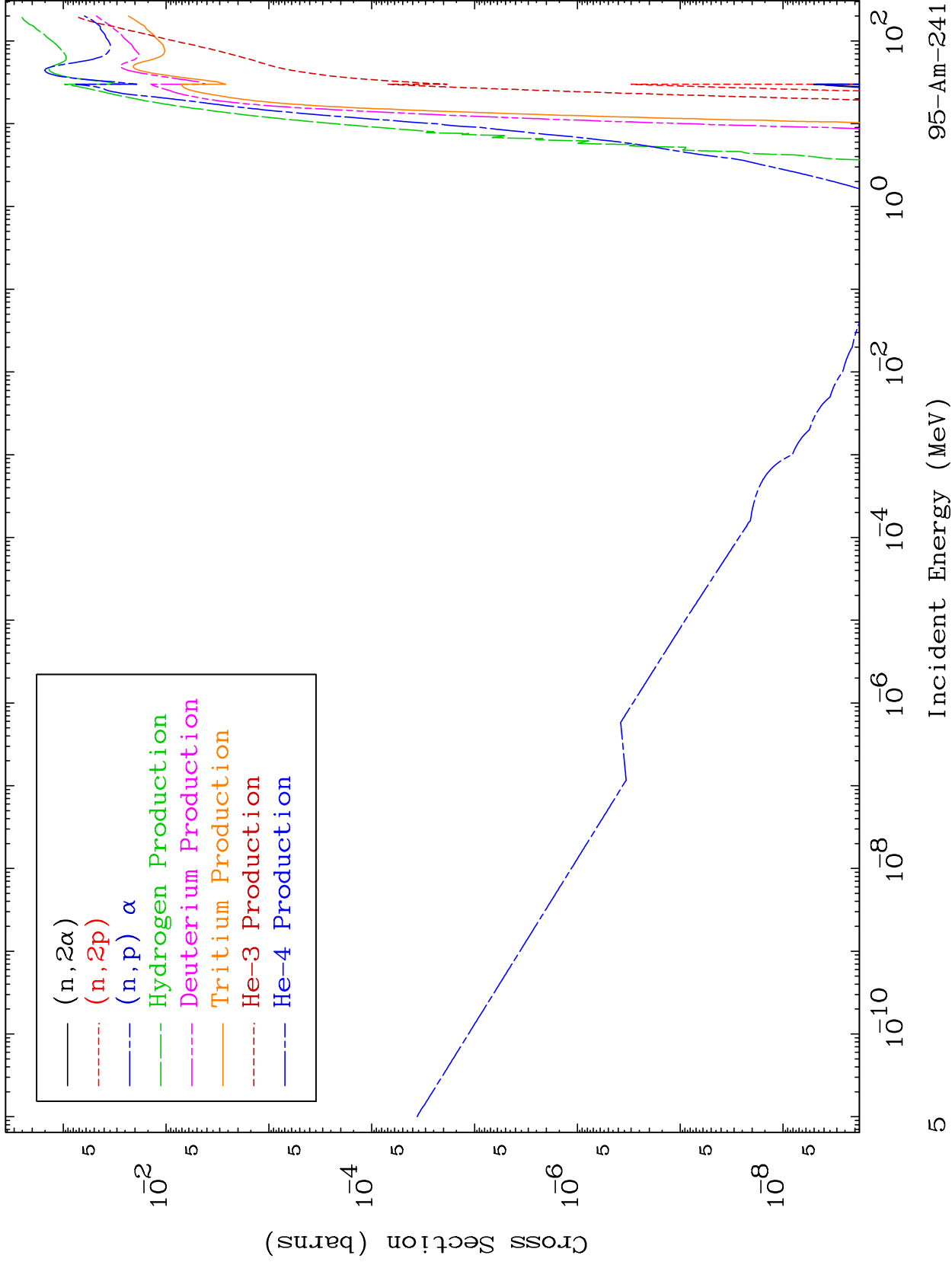
95-Am-241



MAT 9543

Neutron Absorption  
293 Kelvin Cross Sections

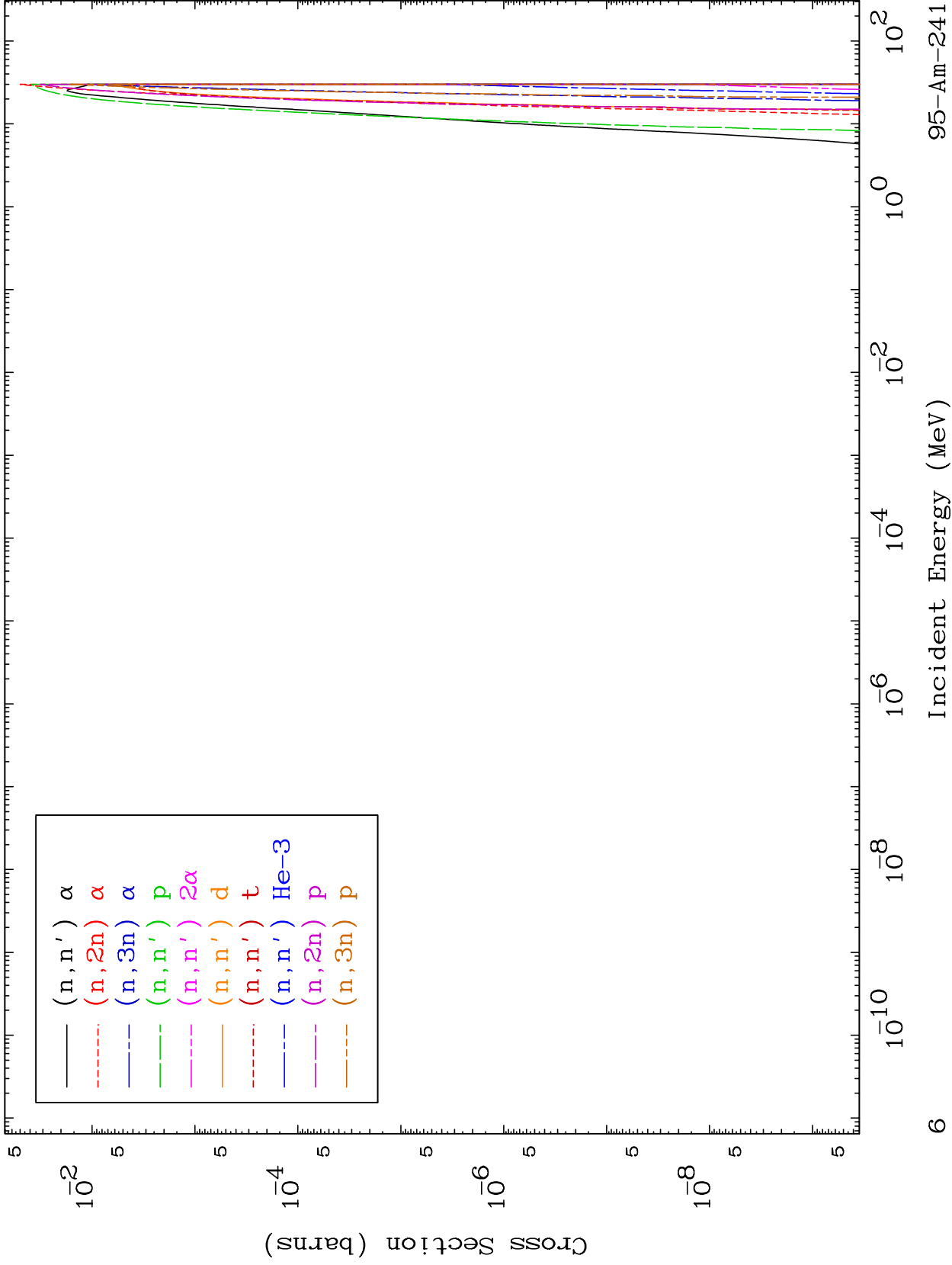
95-Am-241



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Charged Particle  
293 Kelvin Cross Sections

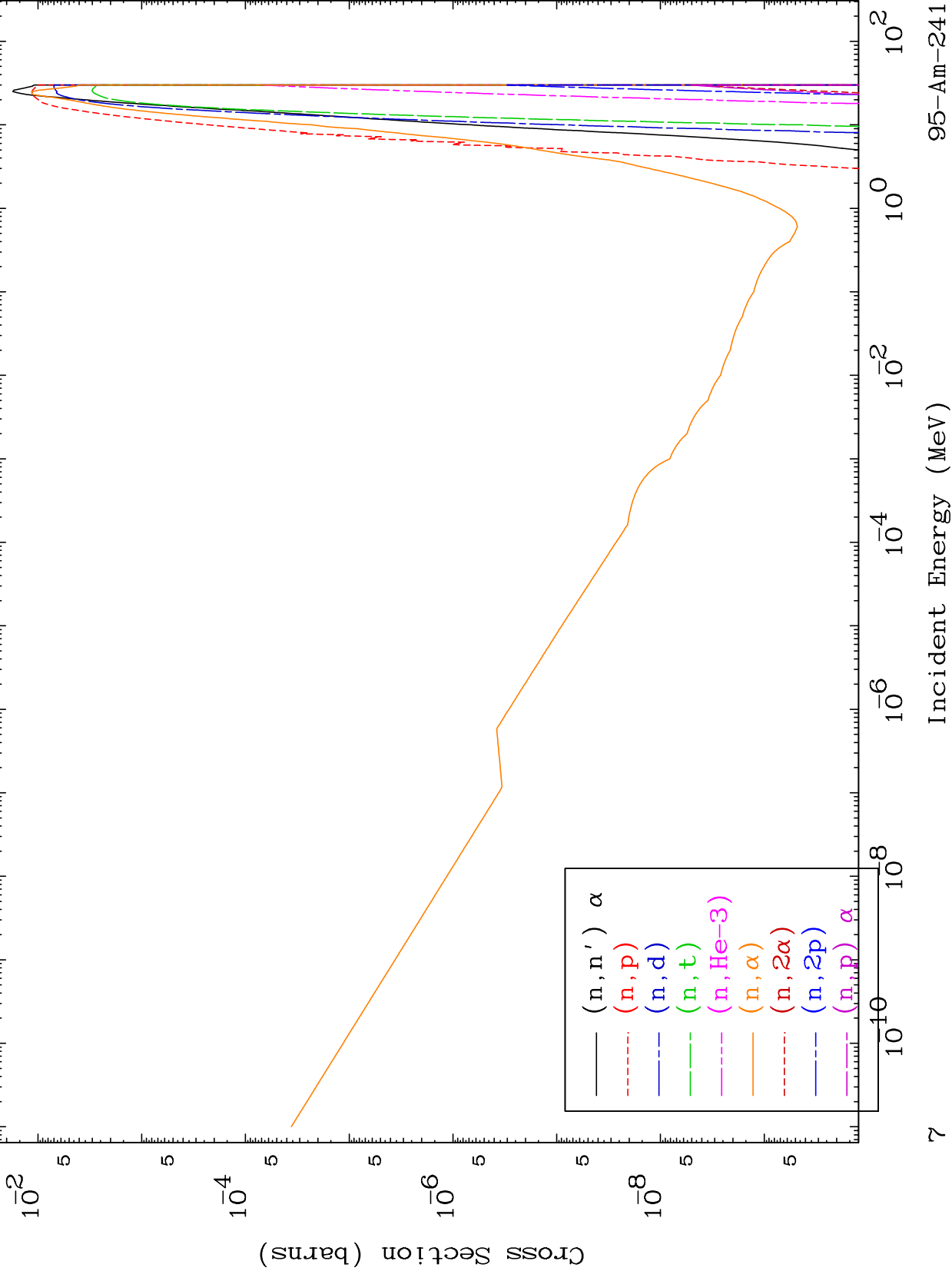
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Charged Particle  
293 Kelvin Cross Sections

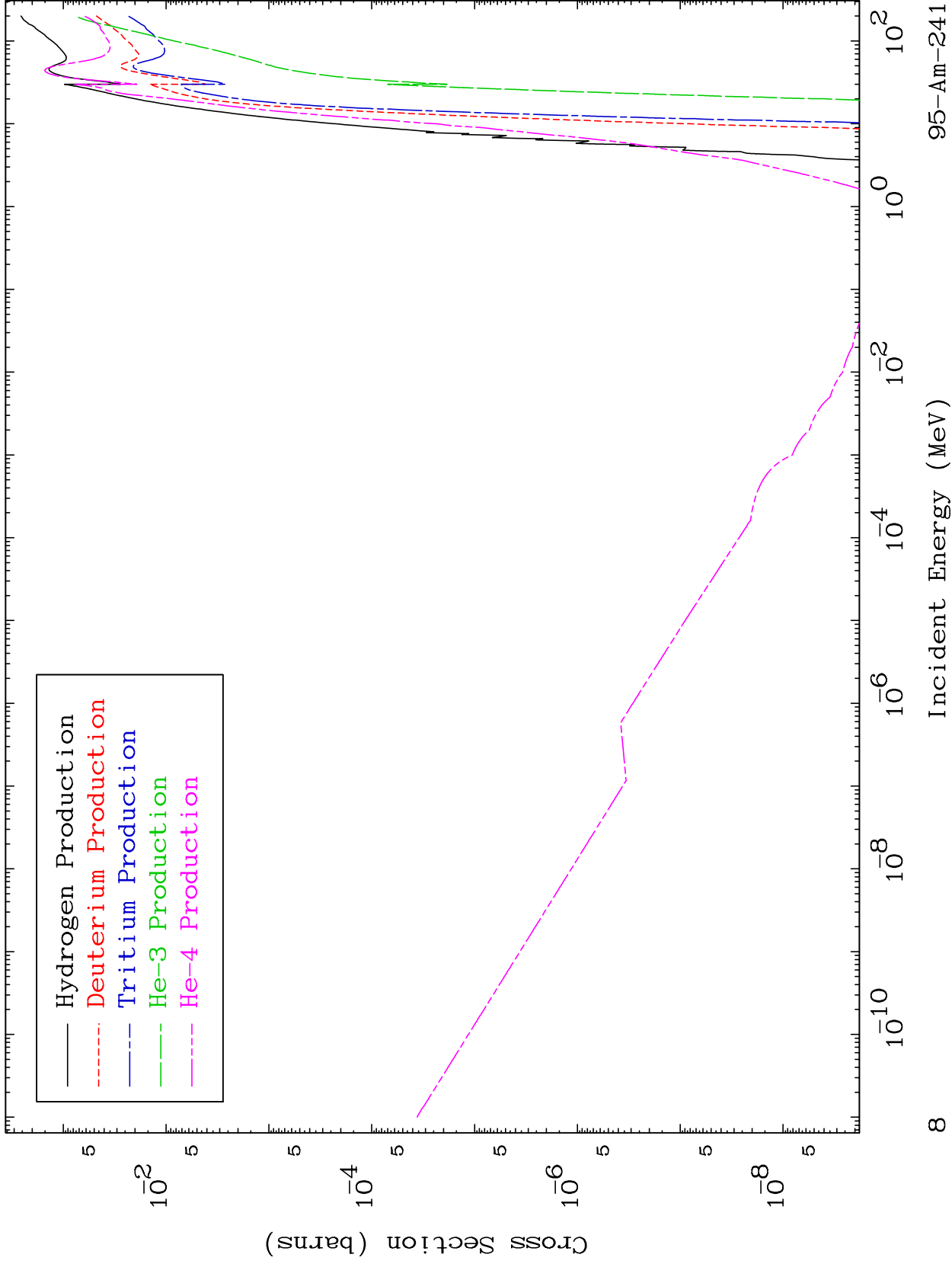
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Particle Production  
293 Kelvin Cross Sections

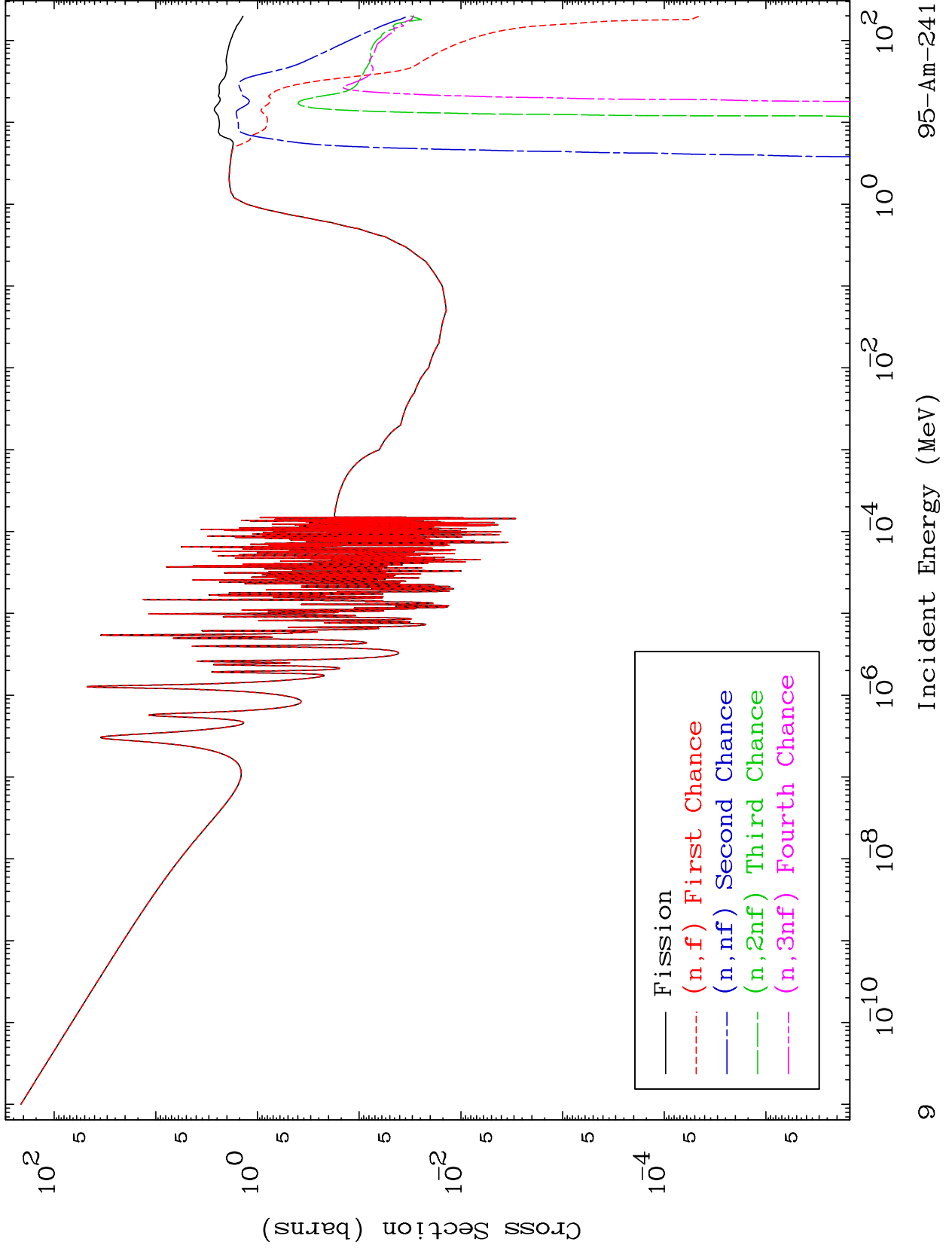
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Fission  
293 Kelvin Cross Sections

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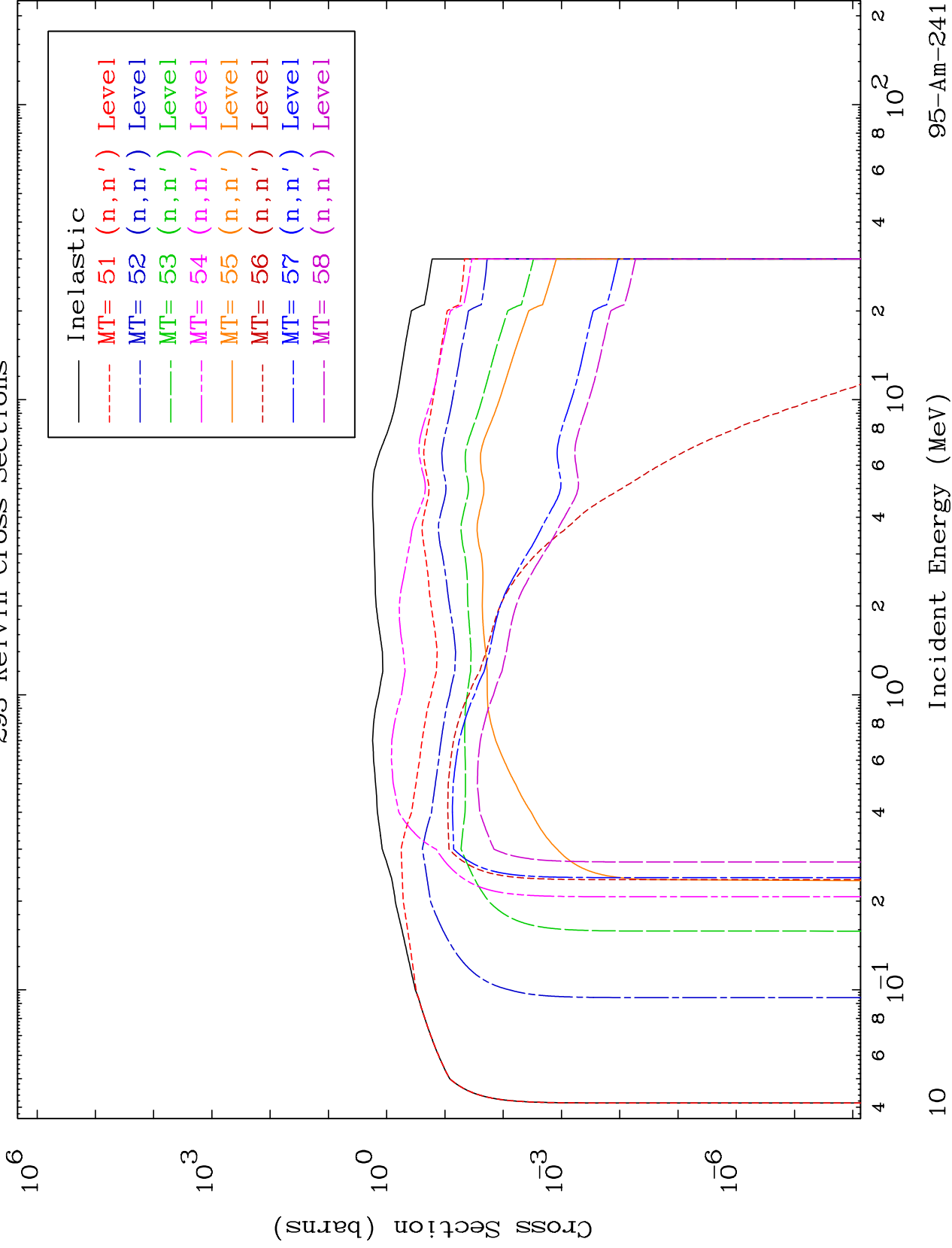


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(n,n') Levels

293 Kelvin Cross Sections

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Incident Energy (MeV)

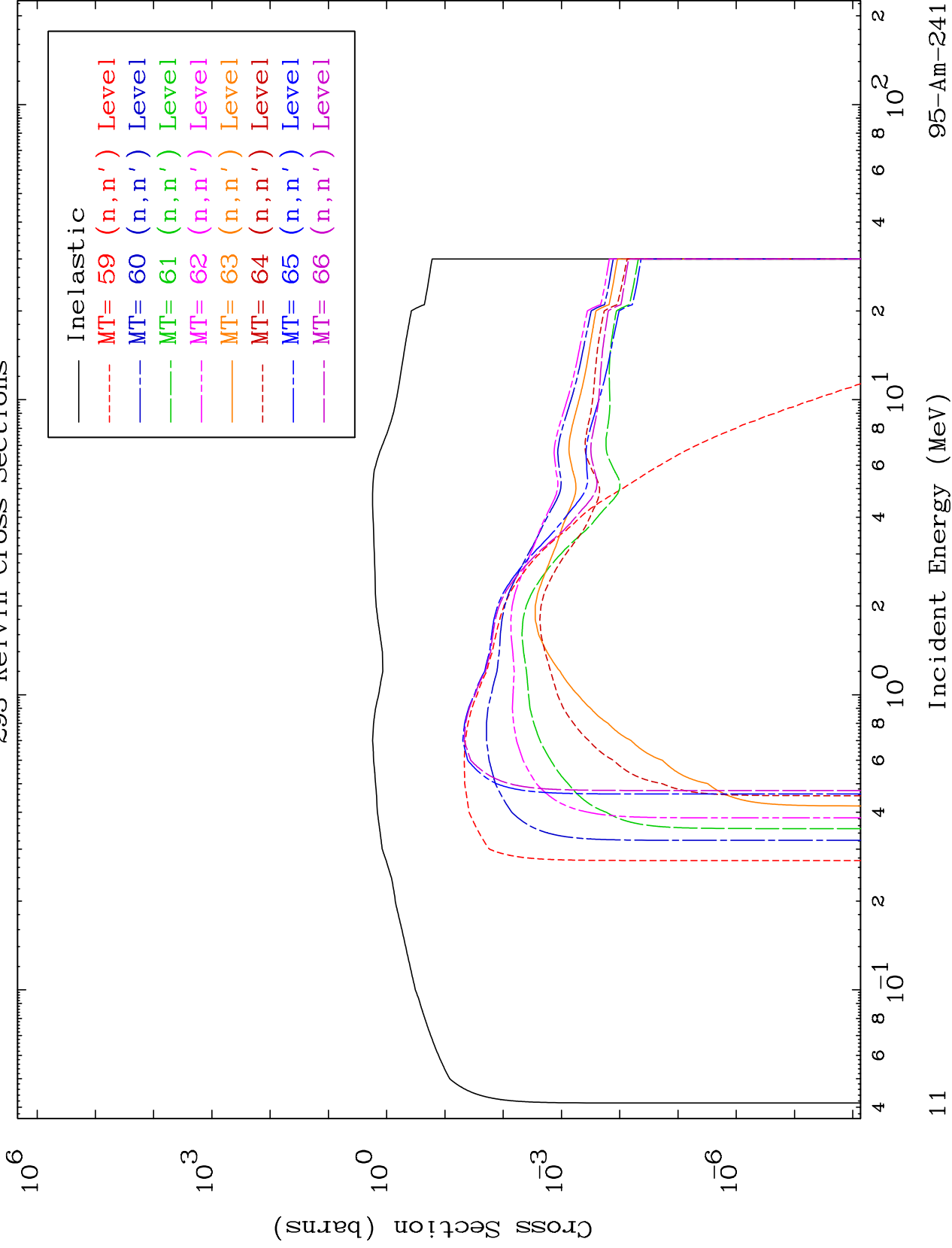
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(n,n') Levels

95-Am-241

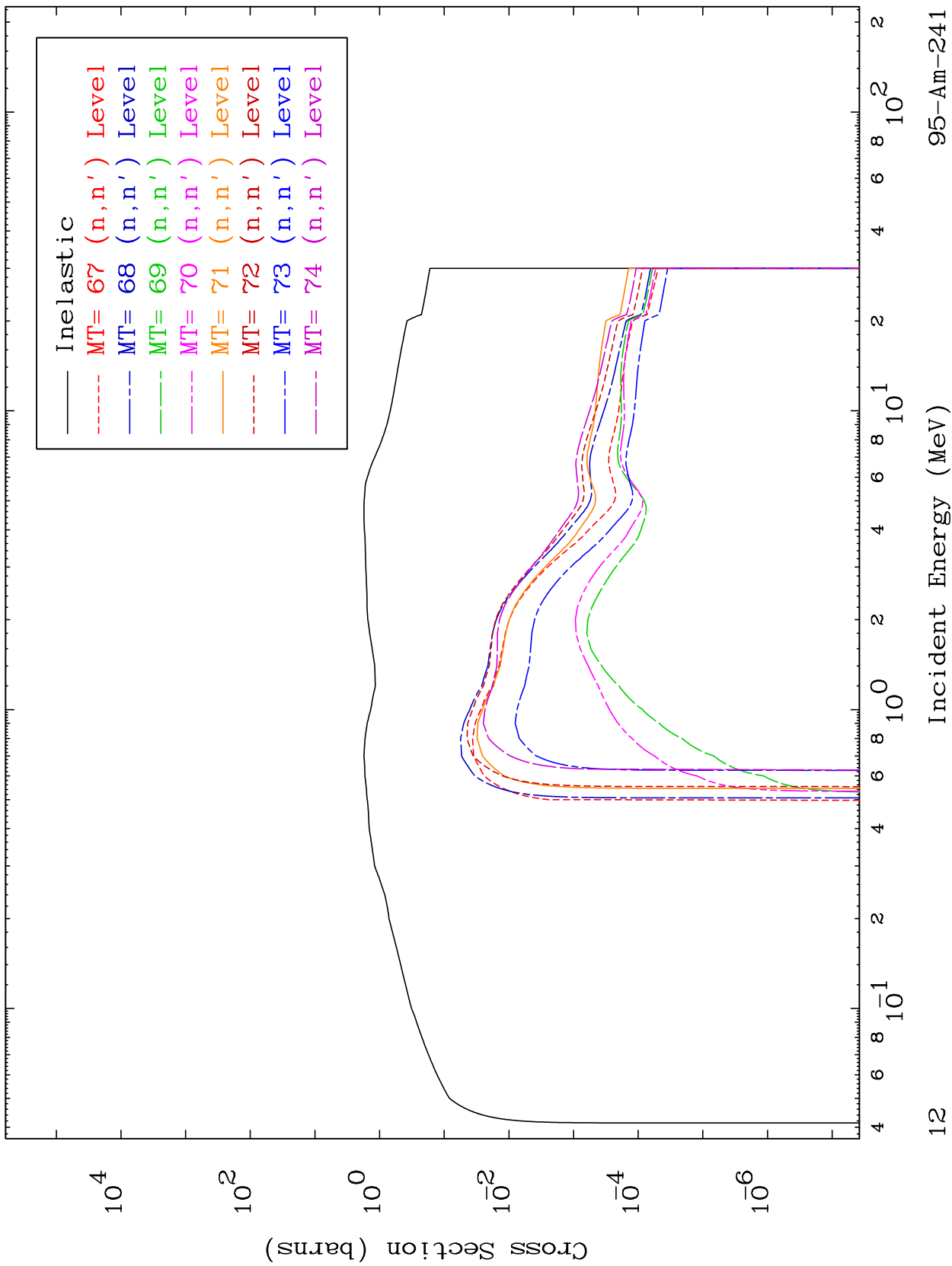
293 Kelvin Cross Sections



MAT 9543

(n,n') Levels  
293 Kelvin Cross Sections

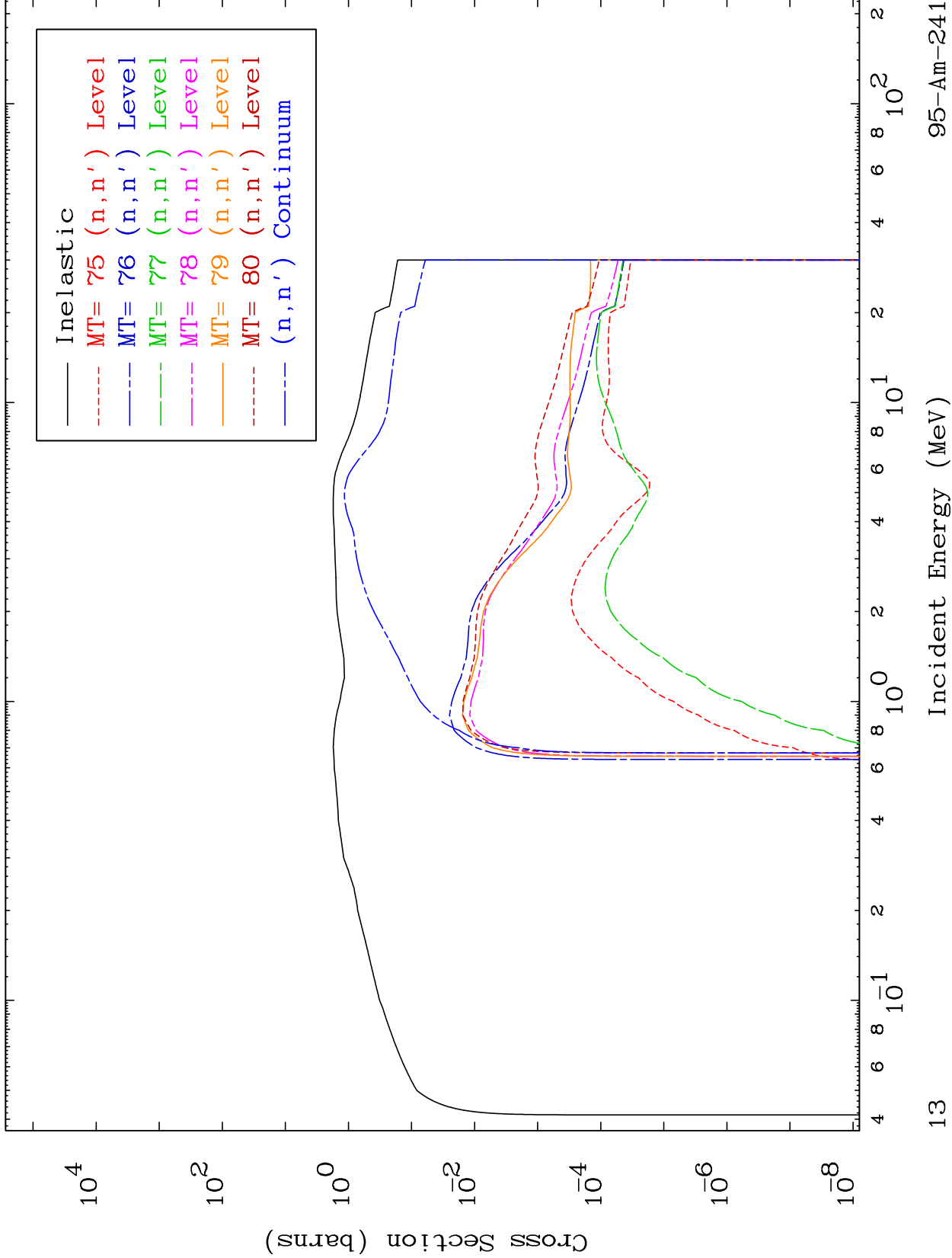
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(n,n') Levels  
293 Kelvin Cross Sections

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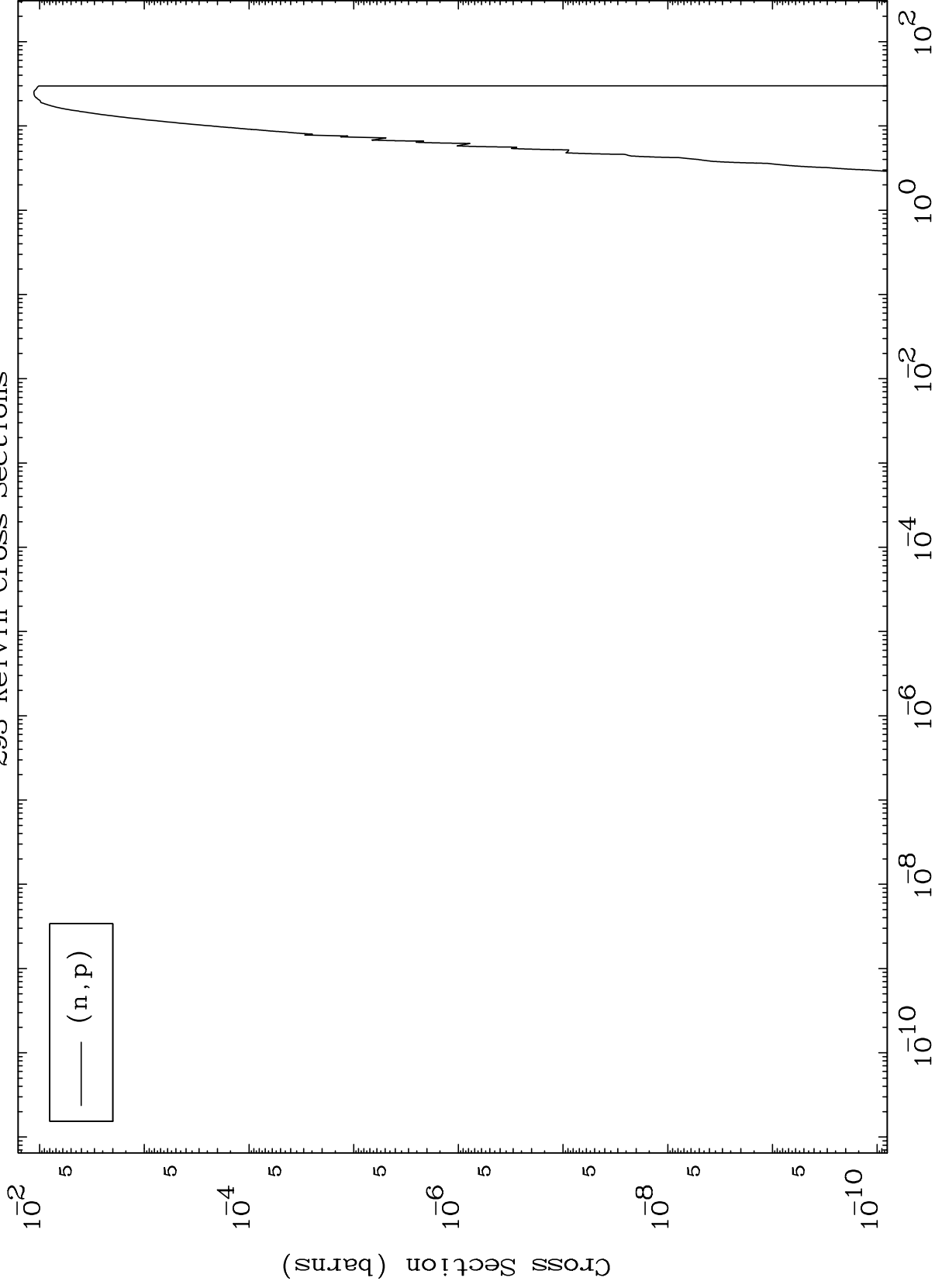
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(n,p) Levels  
293 Kelvin Cross Sections

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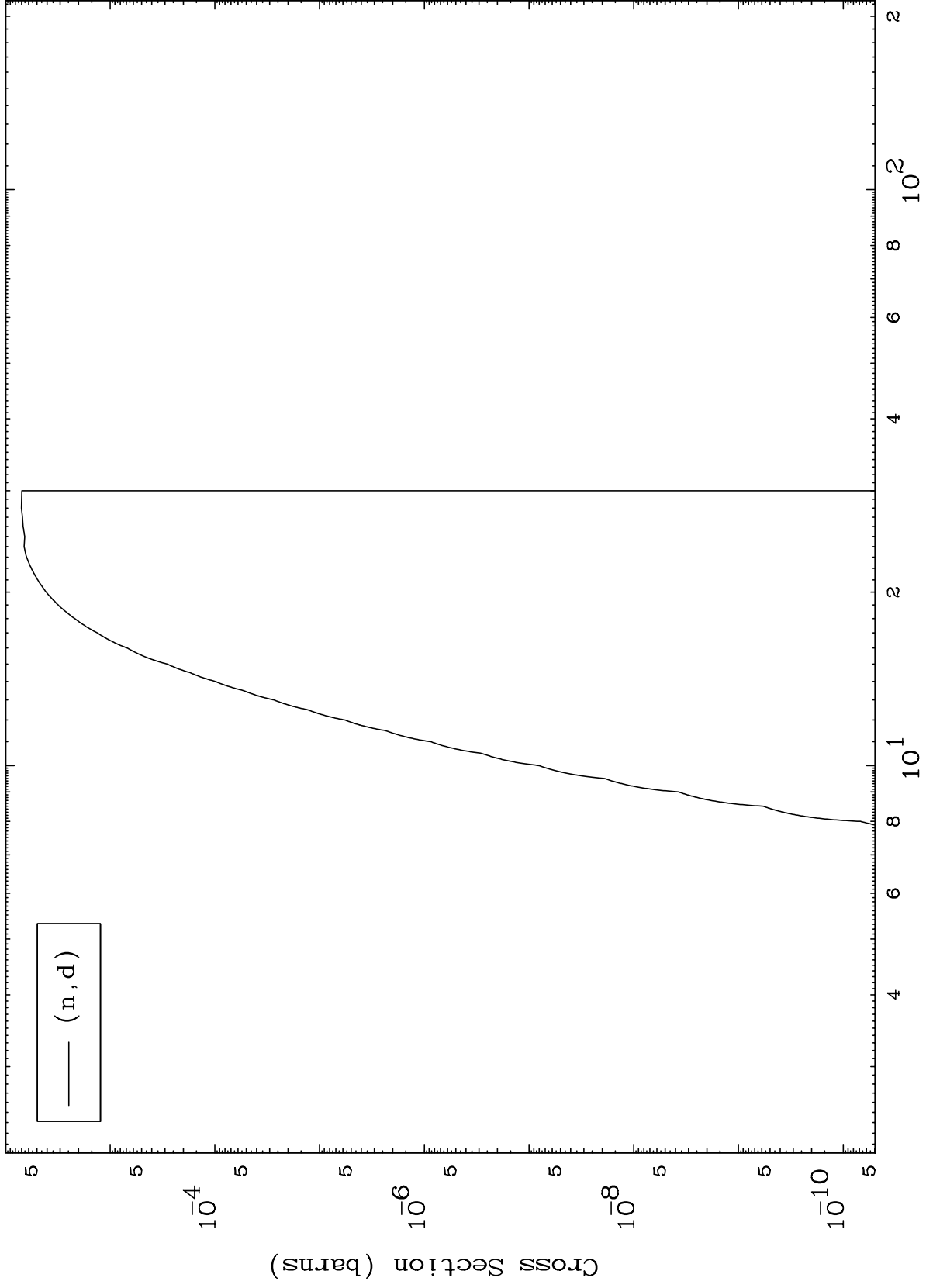
Incident Energy (MeV)

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(n,d) Levels  
293 Kelvin Cross Sections

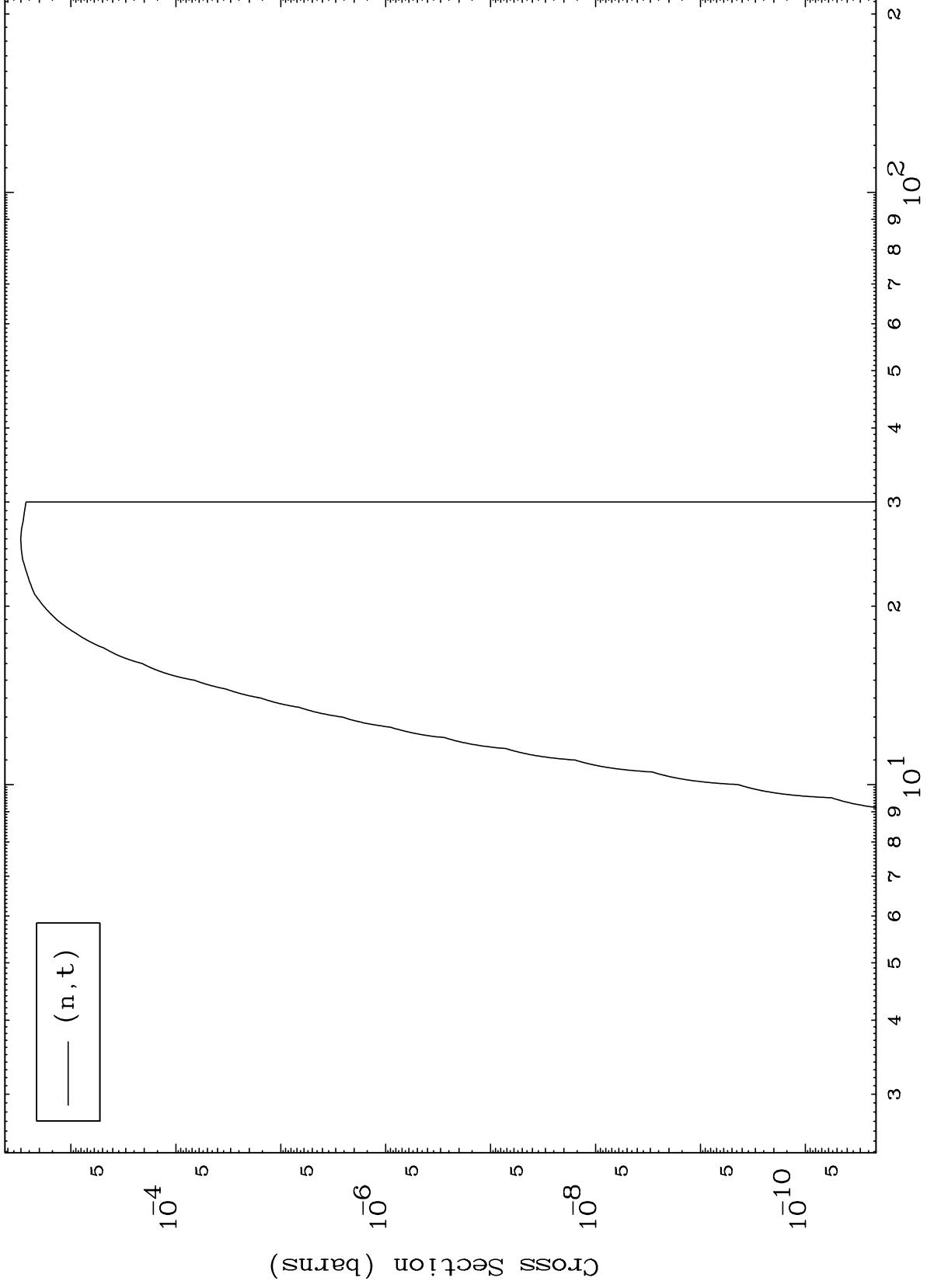
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(n,t) Levels  
293 Kelvin Cross Sections

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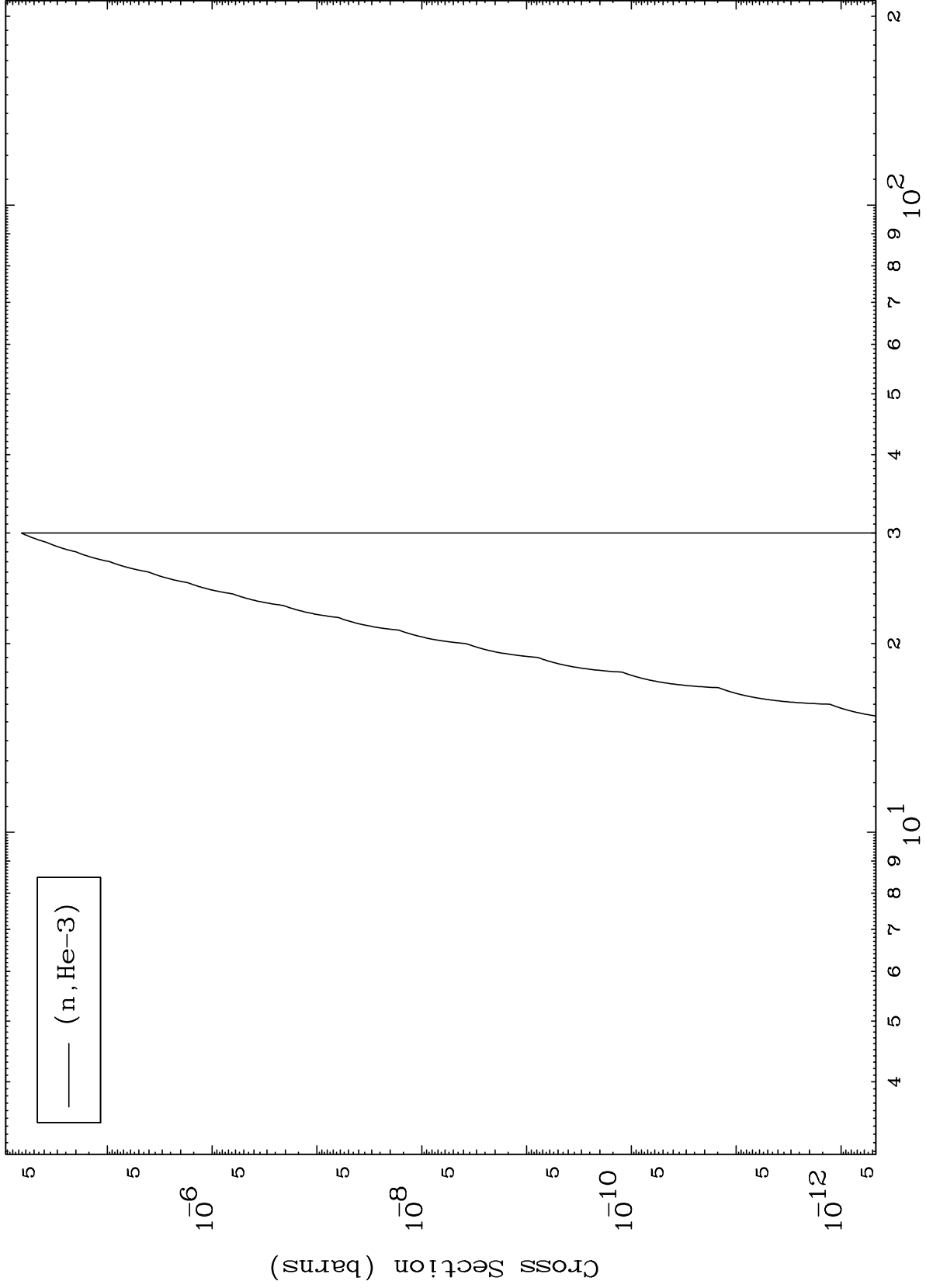
Incident Energy (MeV)

95-Am-241

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(n,He3) Levels  
293 Kelvin Cross Sections

95-Am-241



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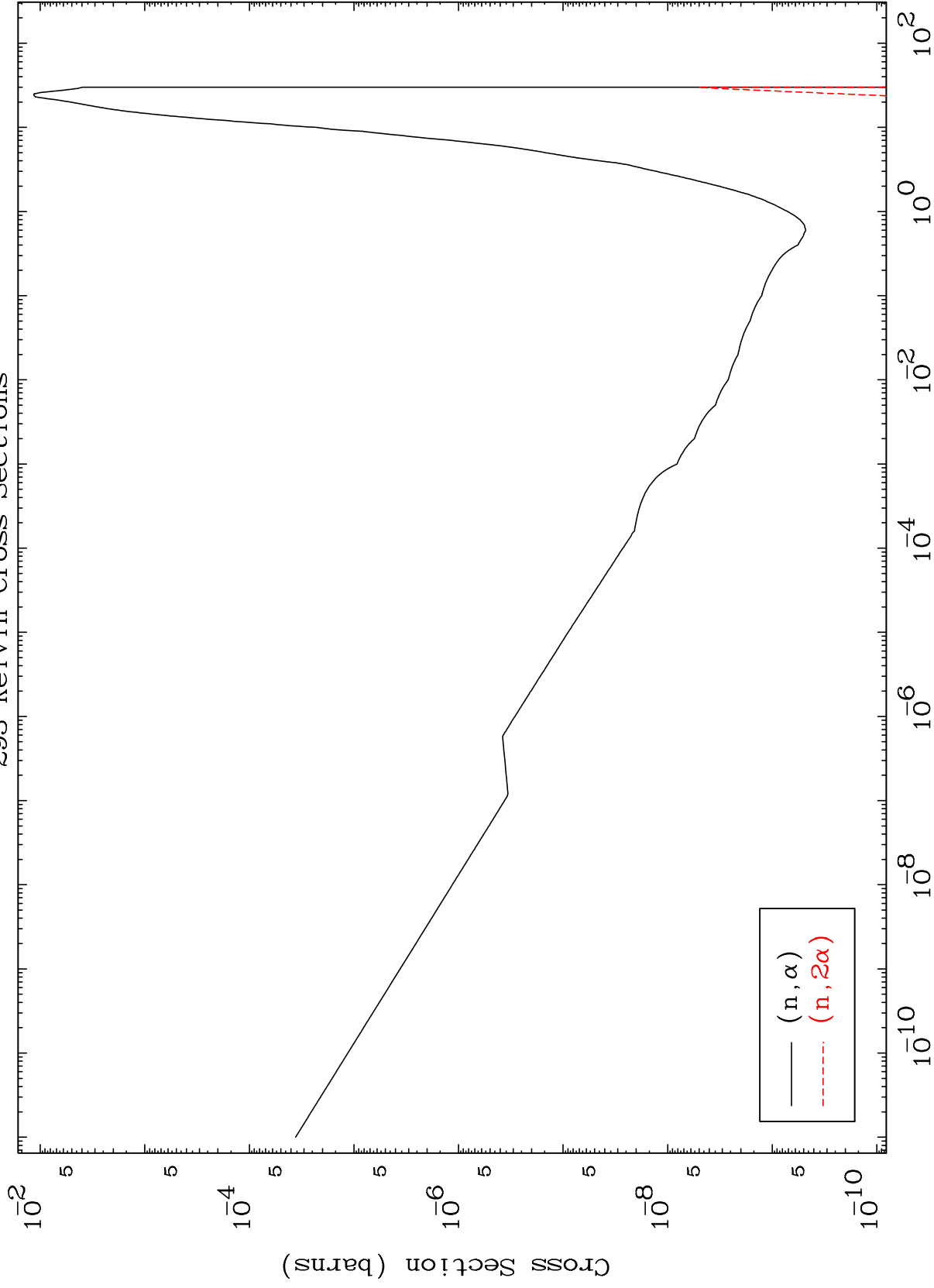
Incident Energy (MeV)

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(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

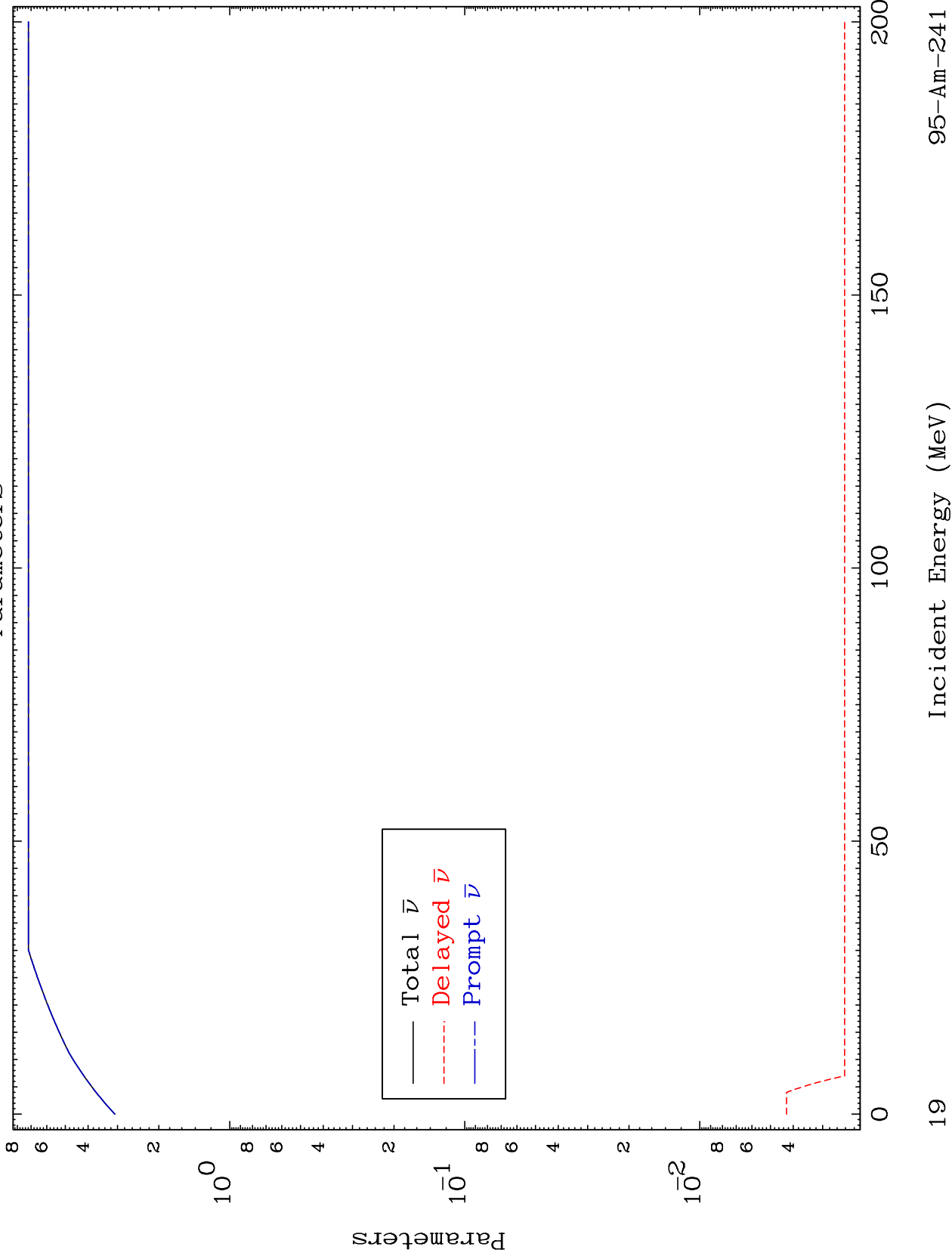
95-Am-241



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Incident Energy (MeV)

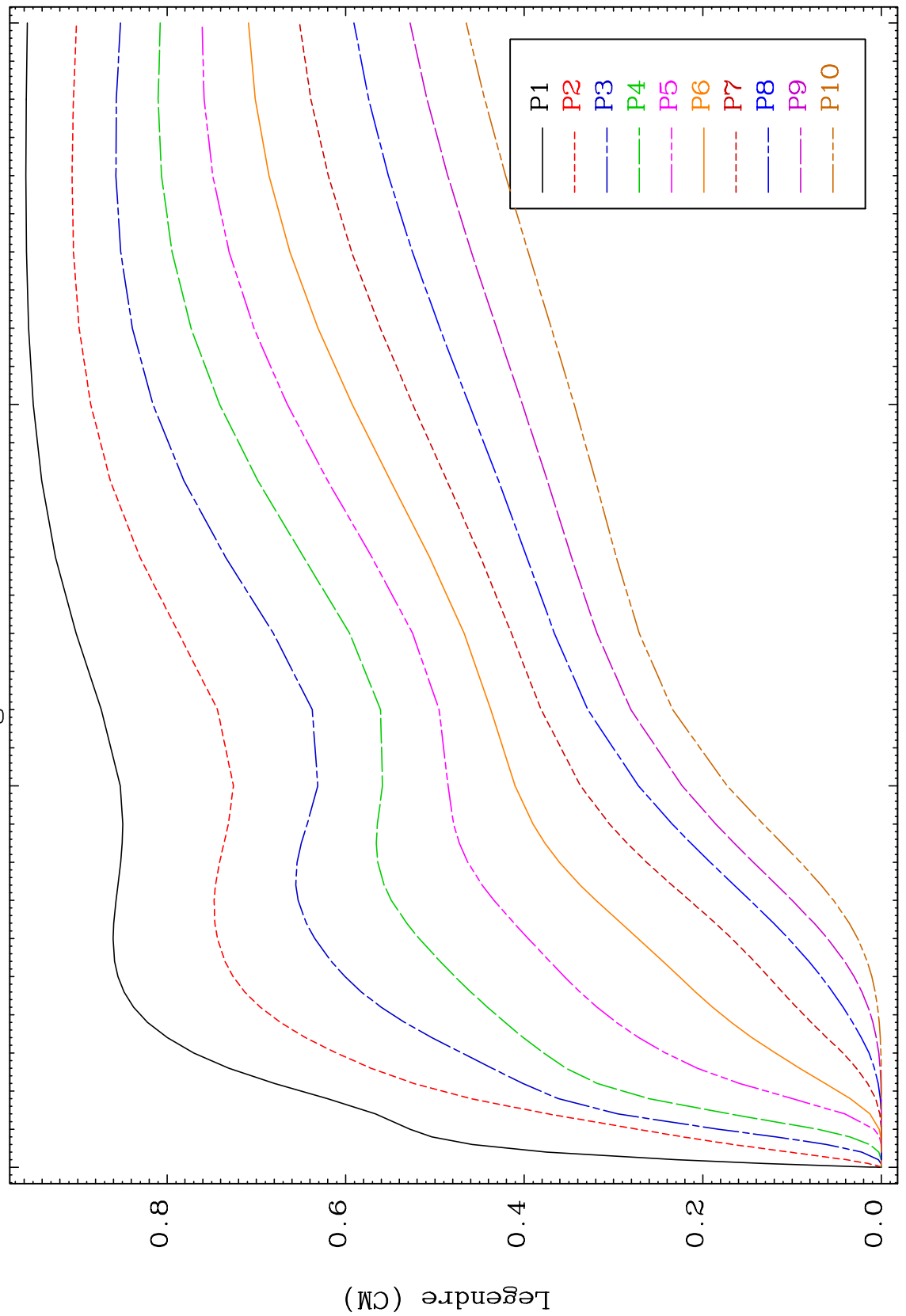
95-Am-241



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Elastic Legendre Coefficients



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Incident Energy (MeV)

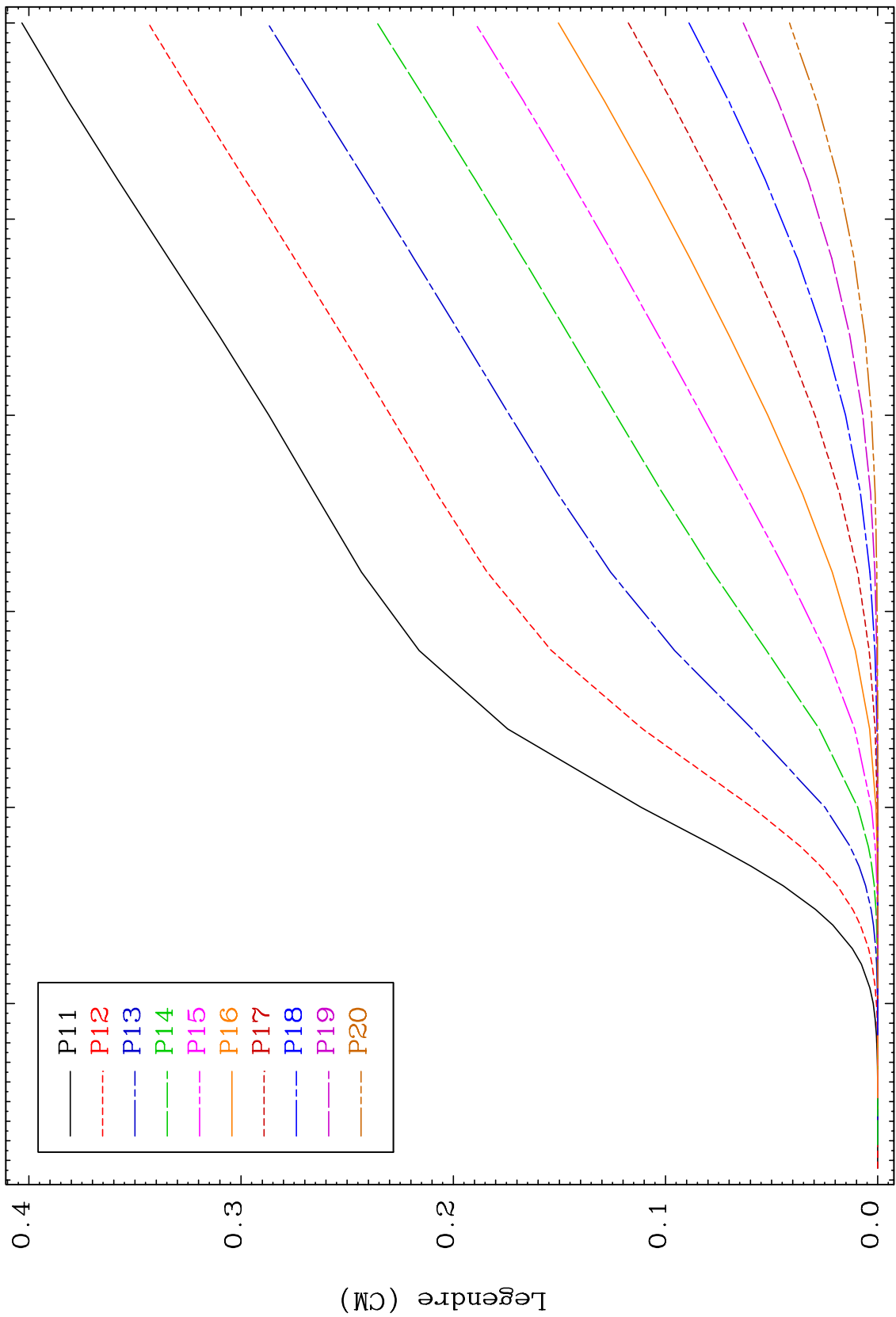
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Elastic Legendre Coefficients

<sup>95</sup>Am-241



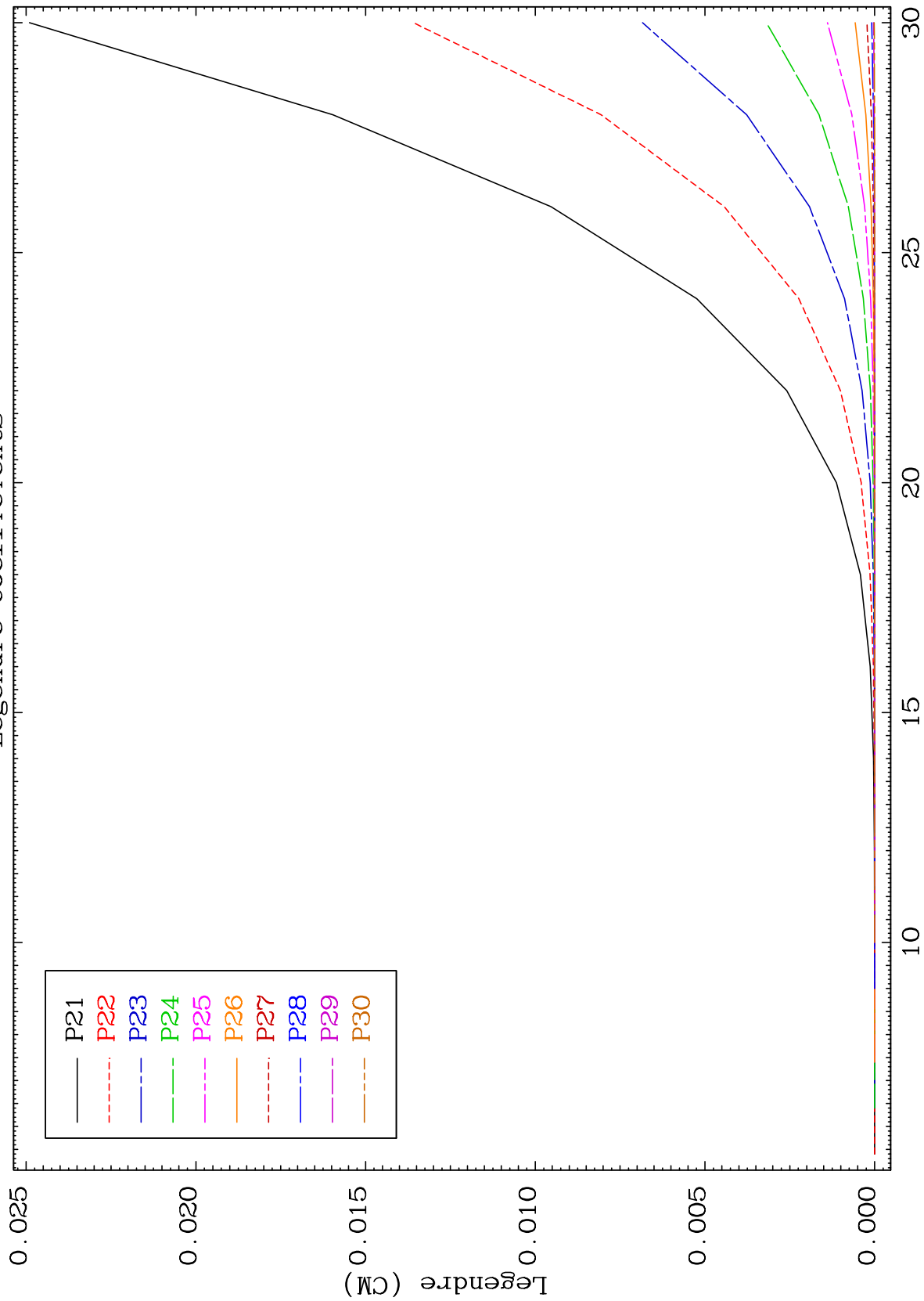
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<sup>95</sup>Am-241

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Elastic  
Legendre Coefficients

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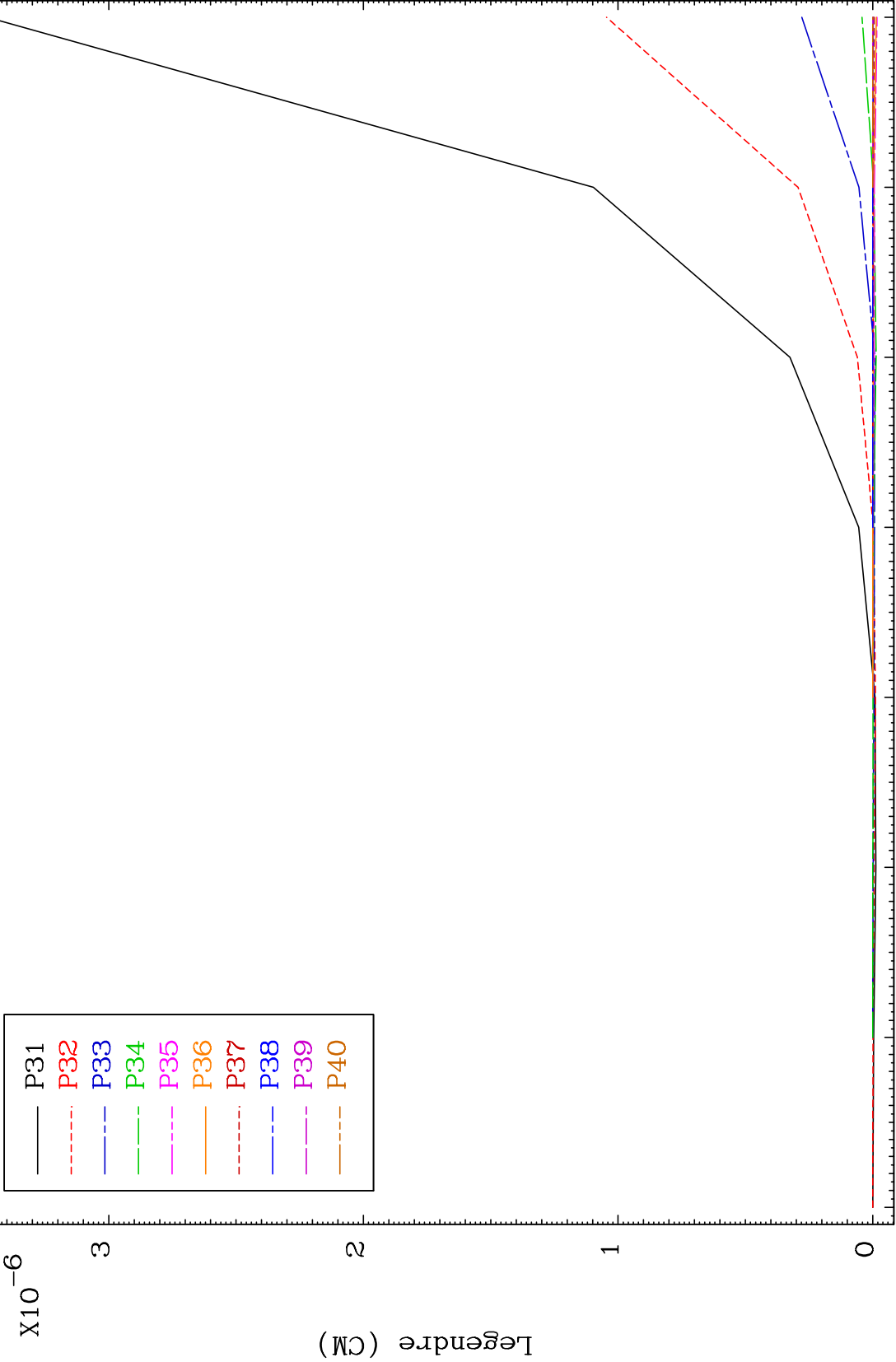
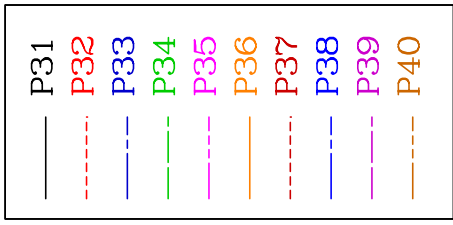
Incident Energy (MeV)

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Elastic Legendre Coefficients

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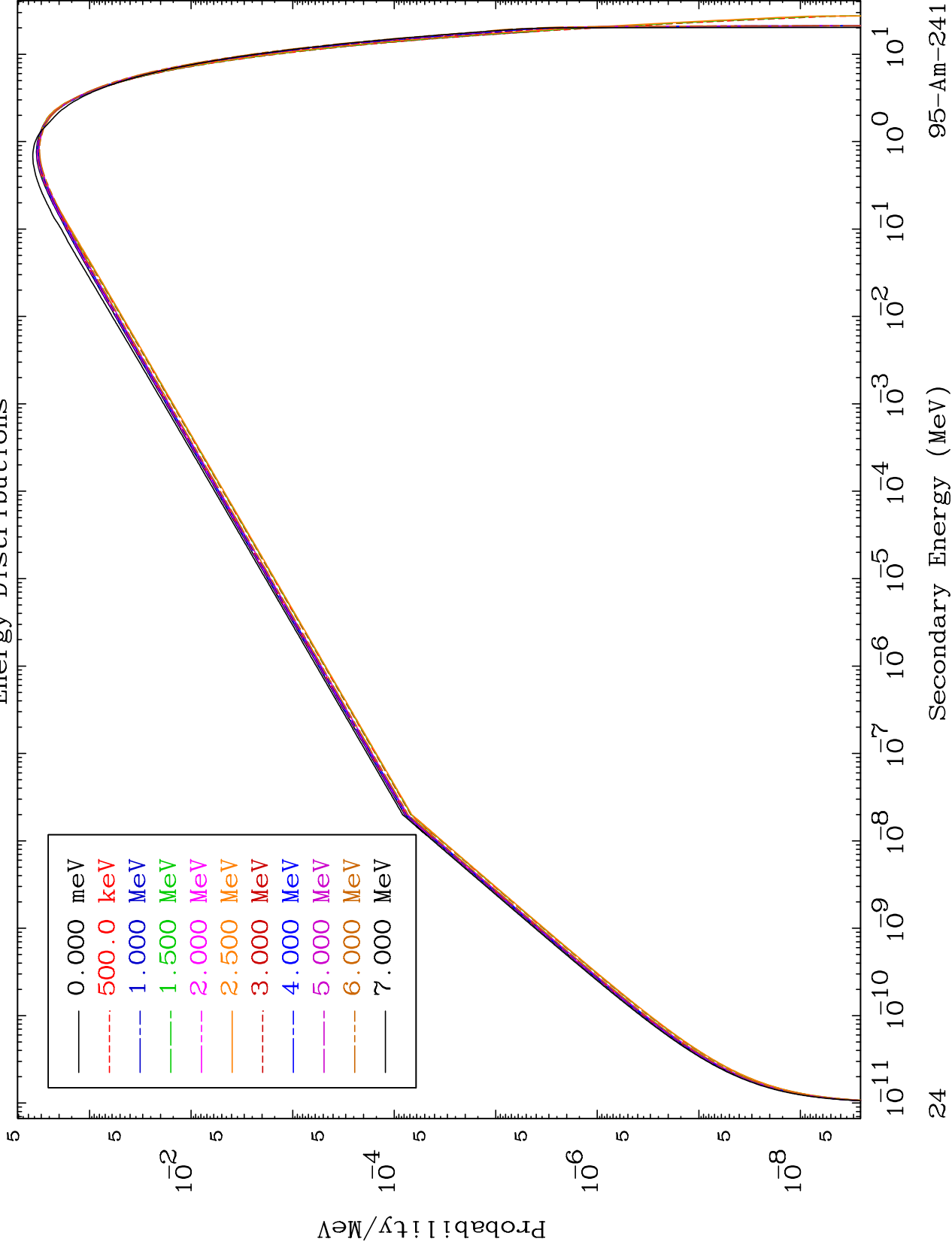
Incident Energy (MeV)

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### Fission Energy Distributions

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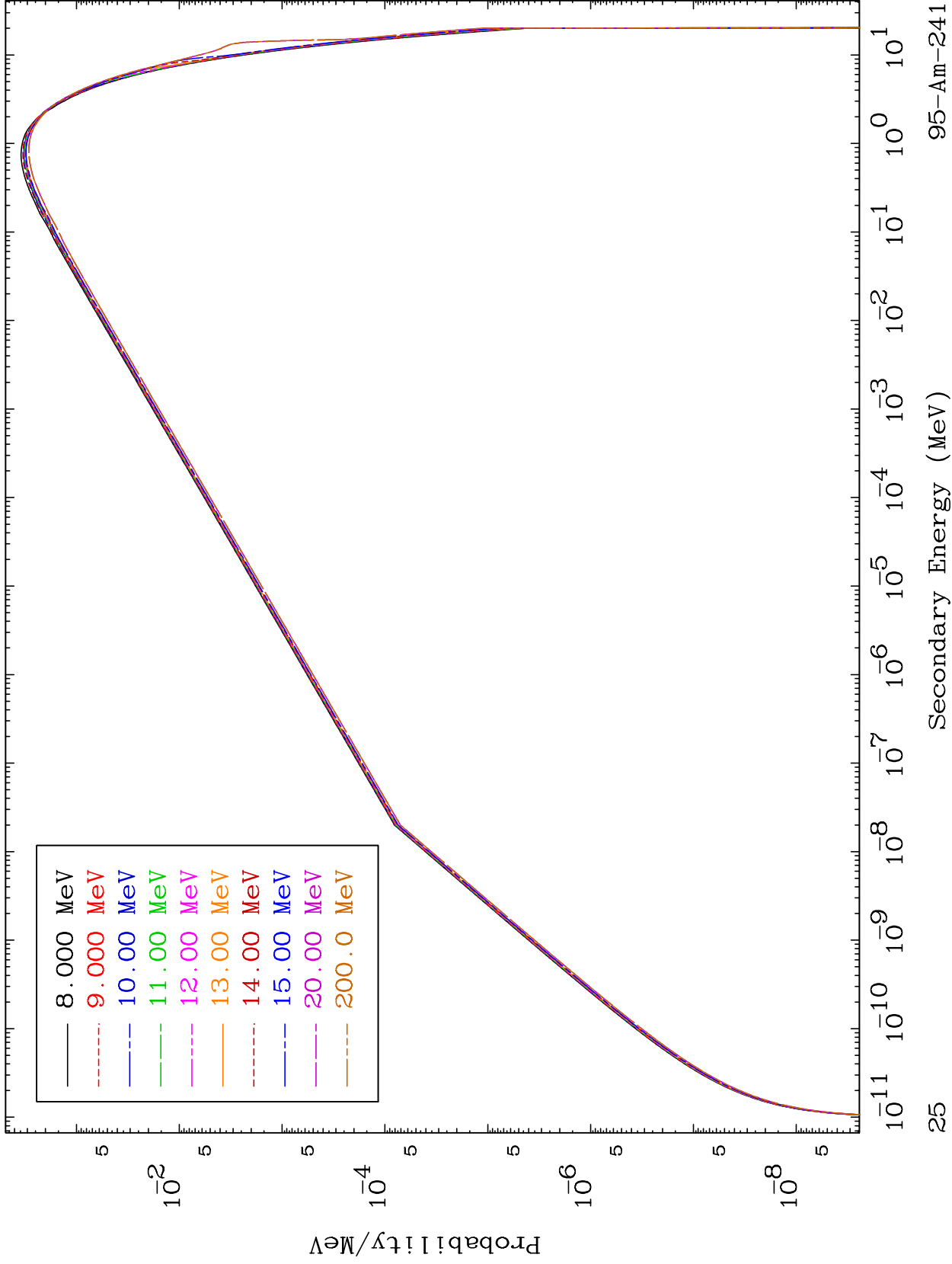


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### Fission Energy Distributions

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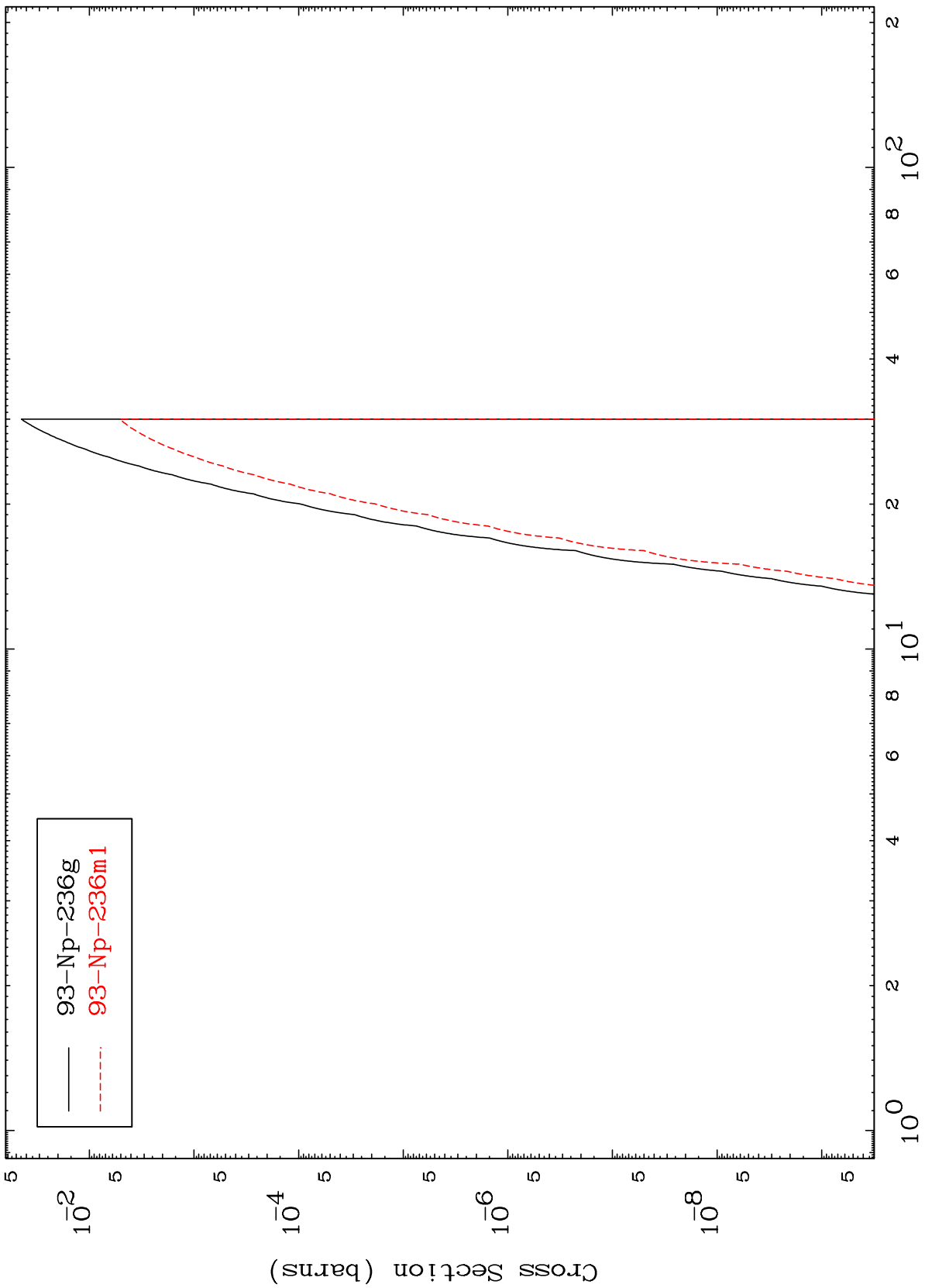
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(n,2n)  $\alpha$

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Radionuclide Production Cross Section



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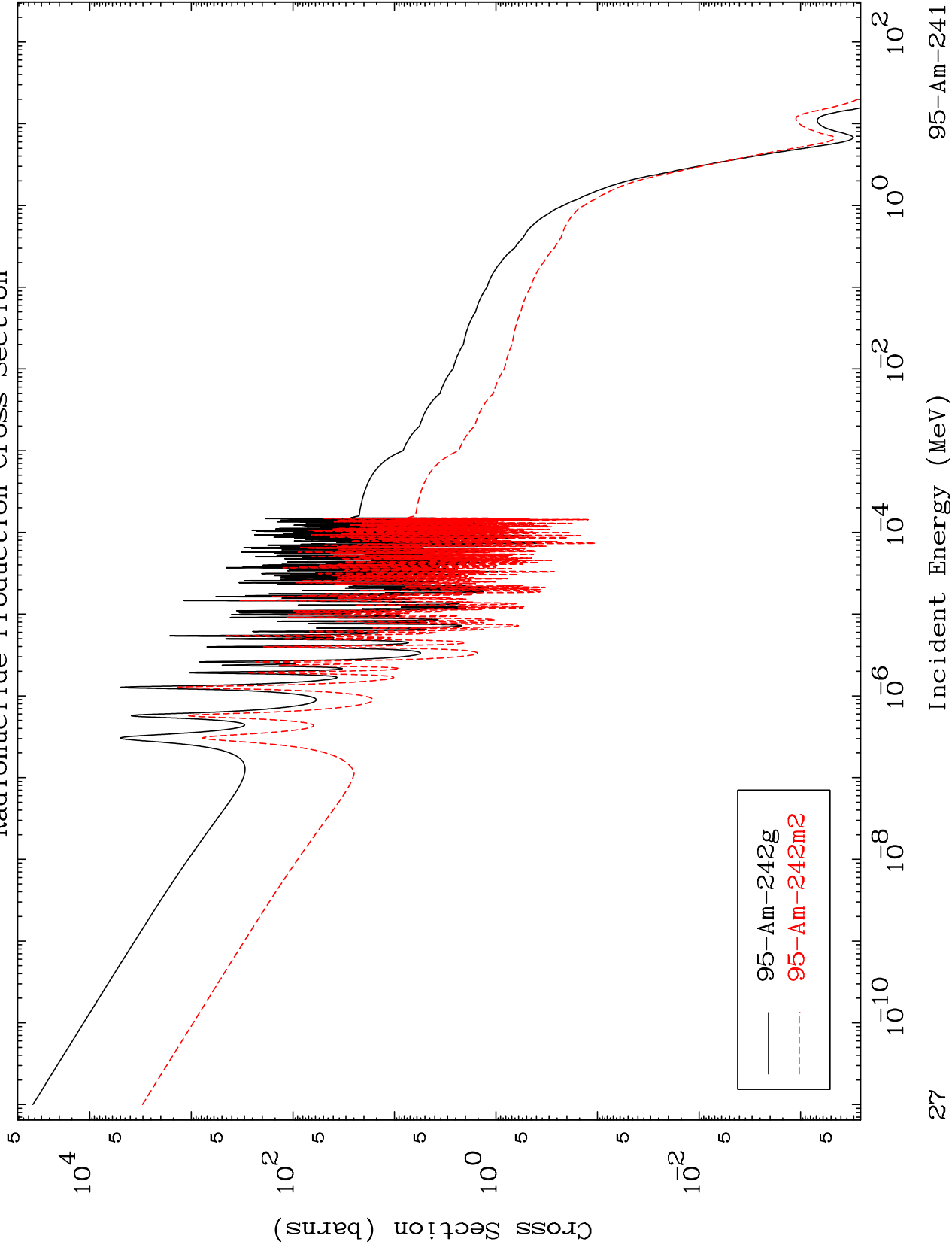
Incident Energy (MeV)

95-Am-241

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Radionuclide Production Cross Section  
(n,  $\gamma$ )

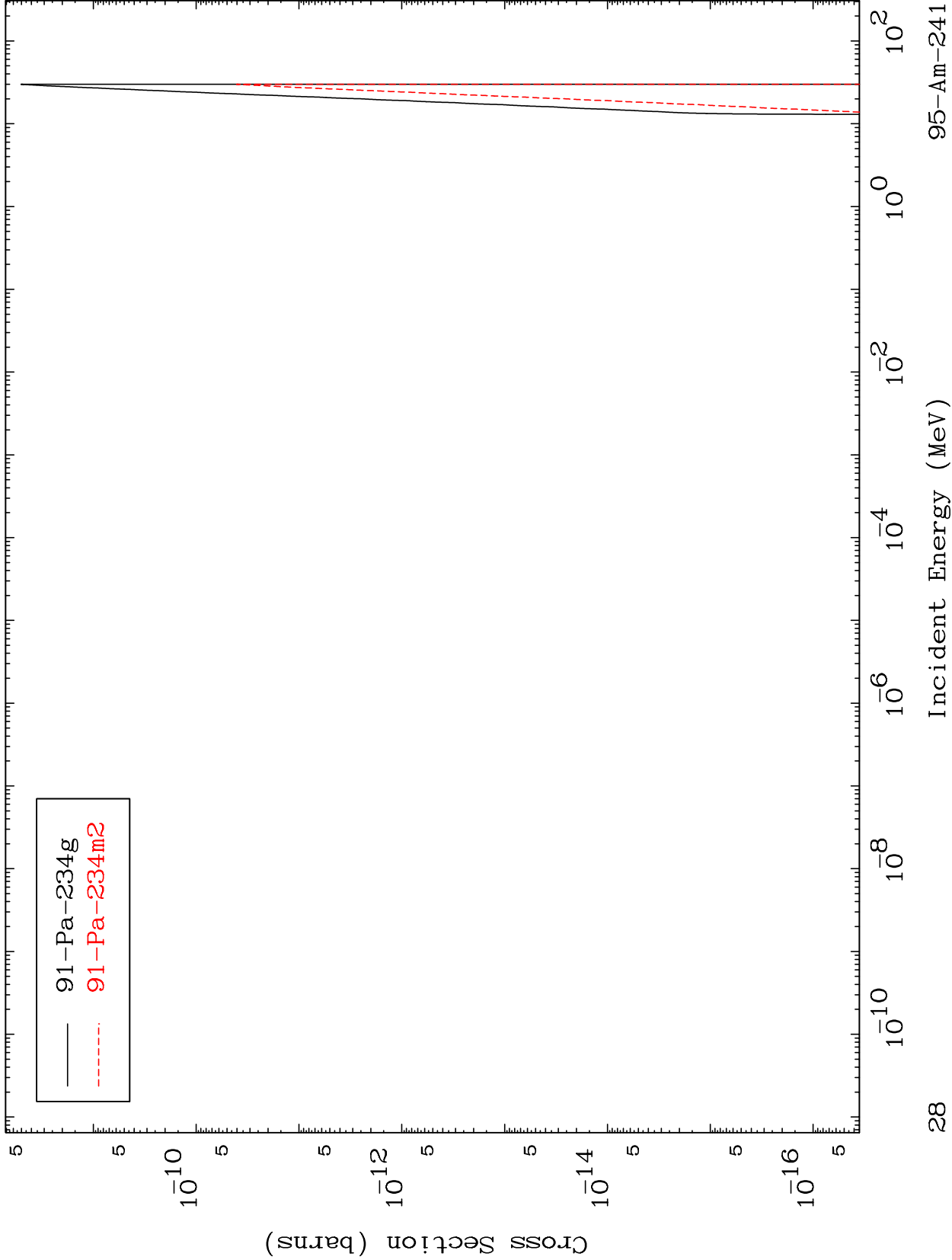


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(n,2α)

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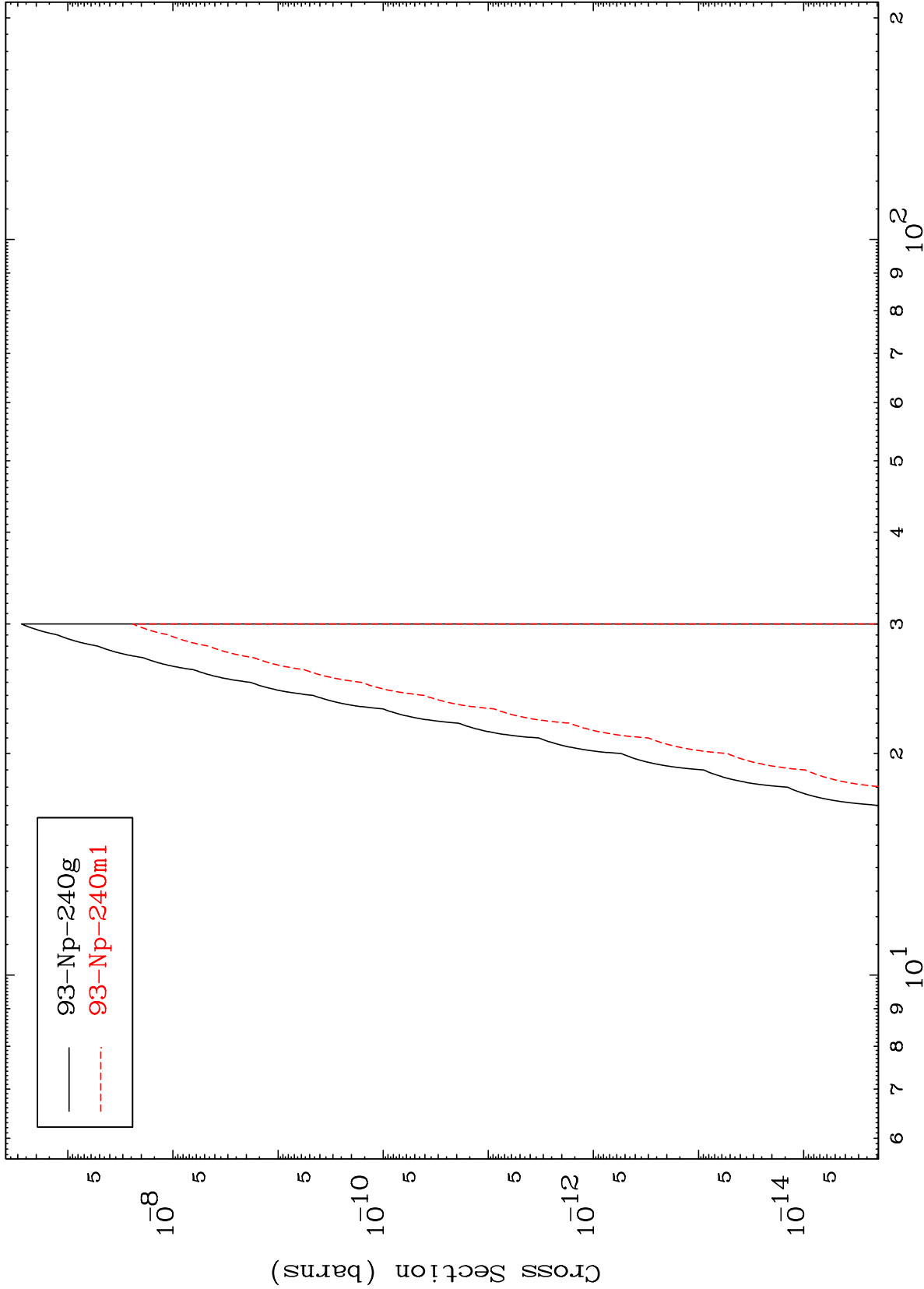
Radionuclide Production Cross Section



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(n,2p)  
Radionuclide Production Cross Section



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Incident Energy (MeV)