

Program EVALPLOT
(Version 2021-1)

by

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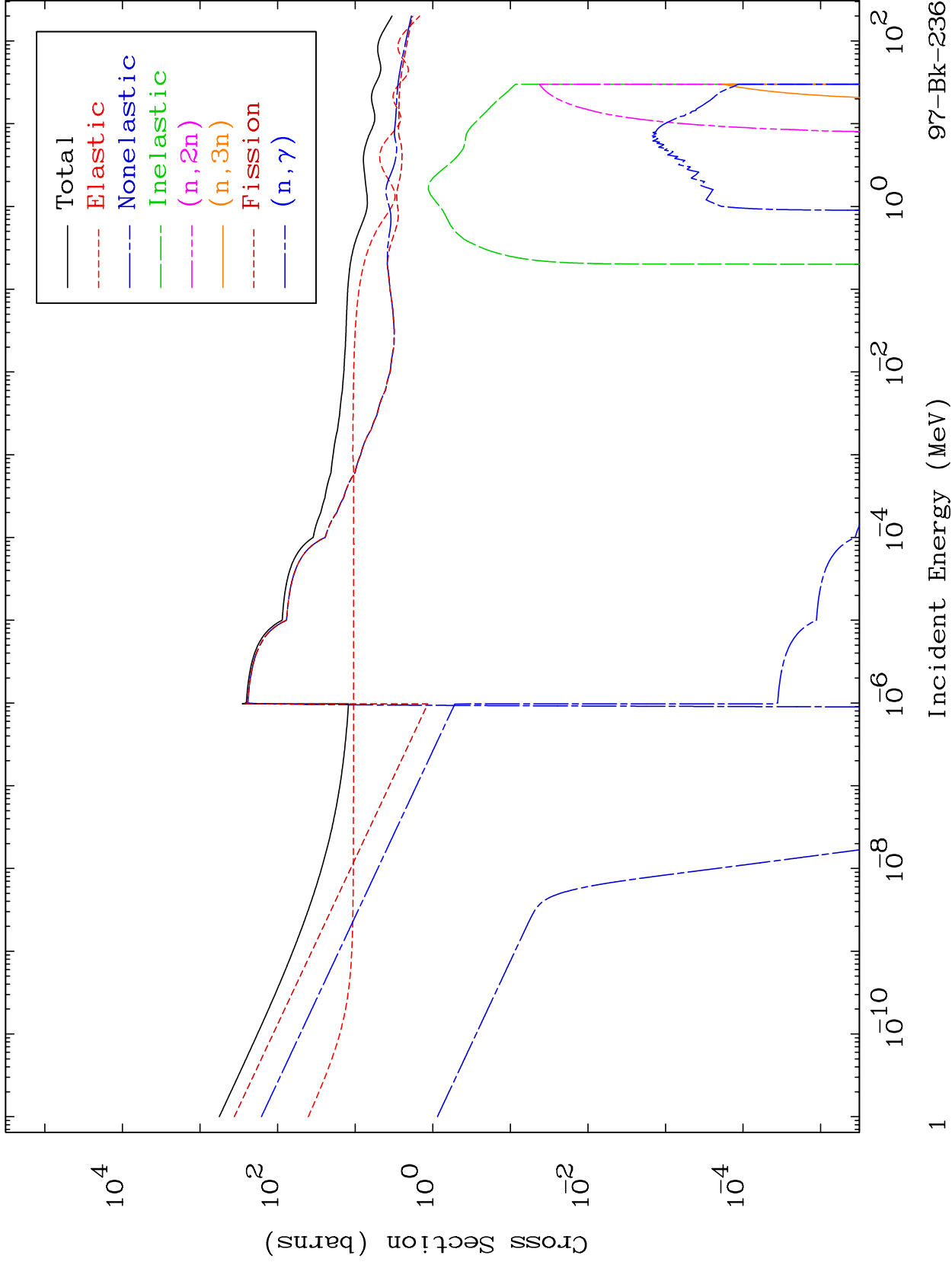
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Press Mouse Button to Start

MAT 9713

Neutron Major
293 Kelvin Cross Sections

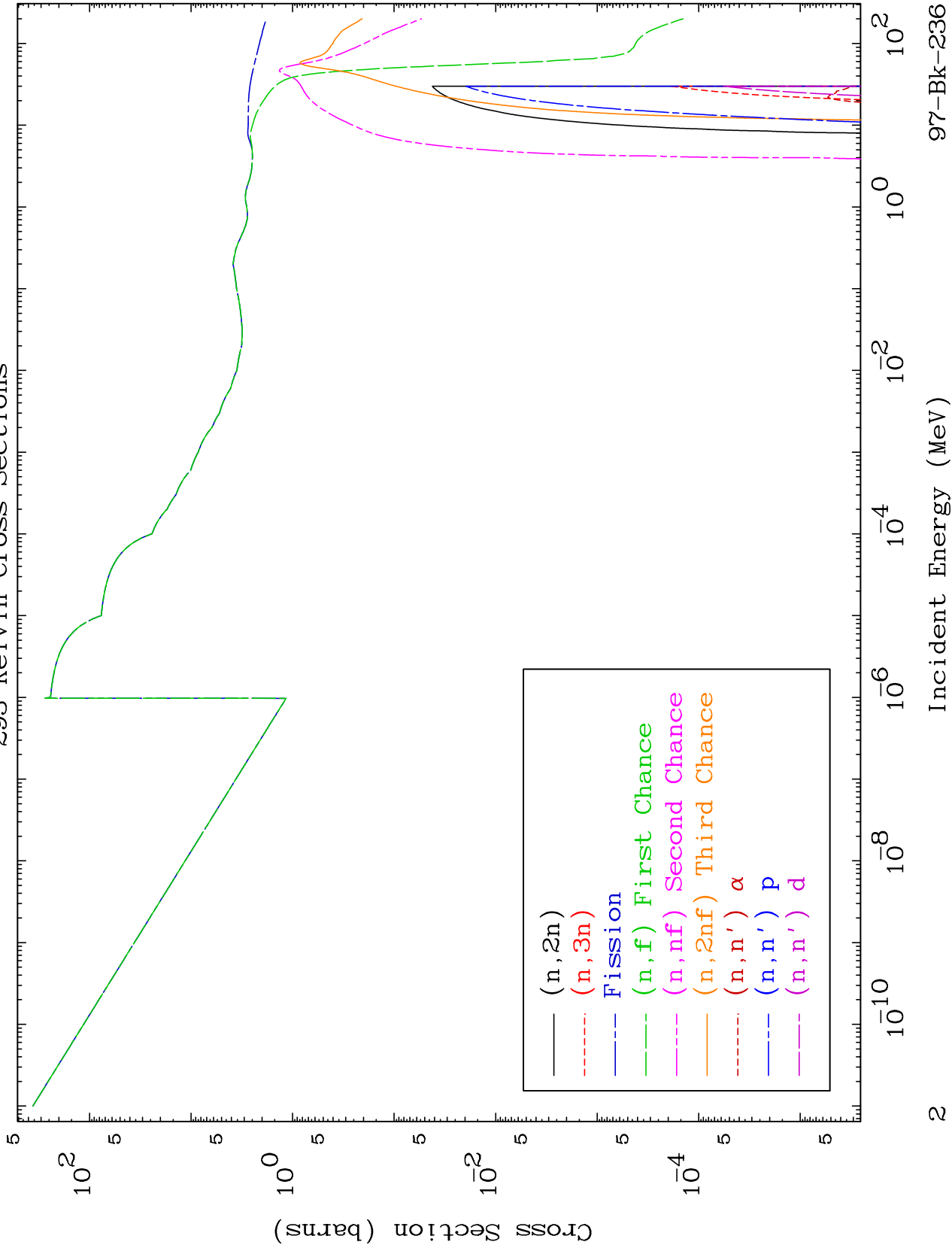
97-Bk-236

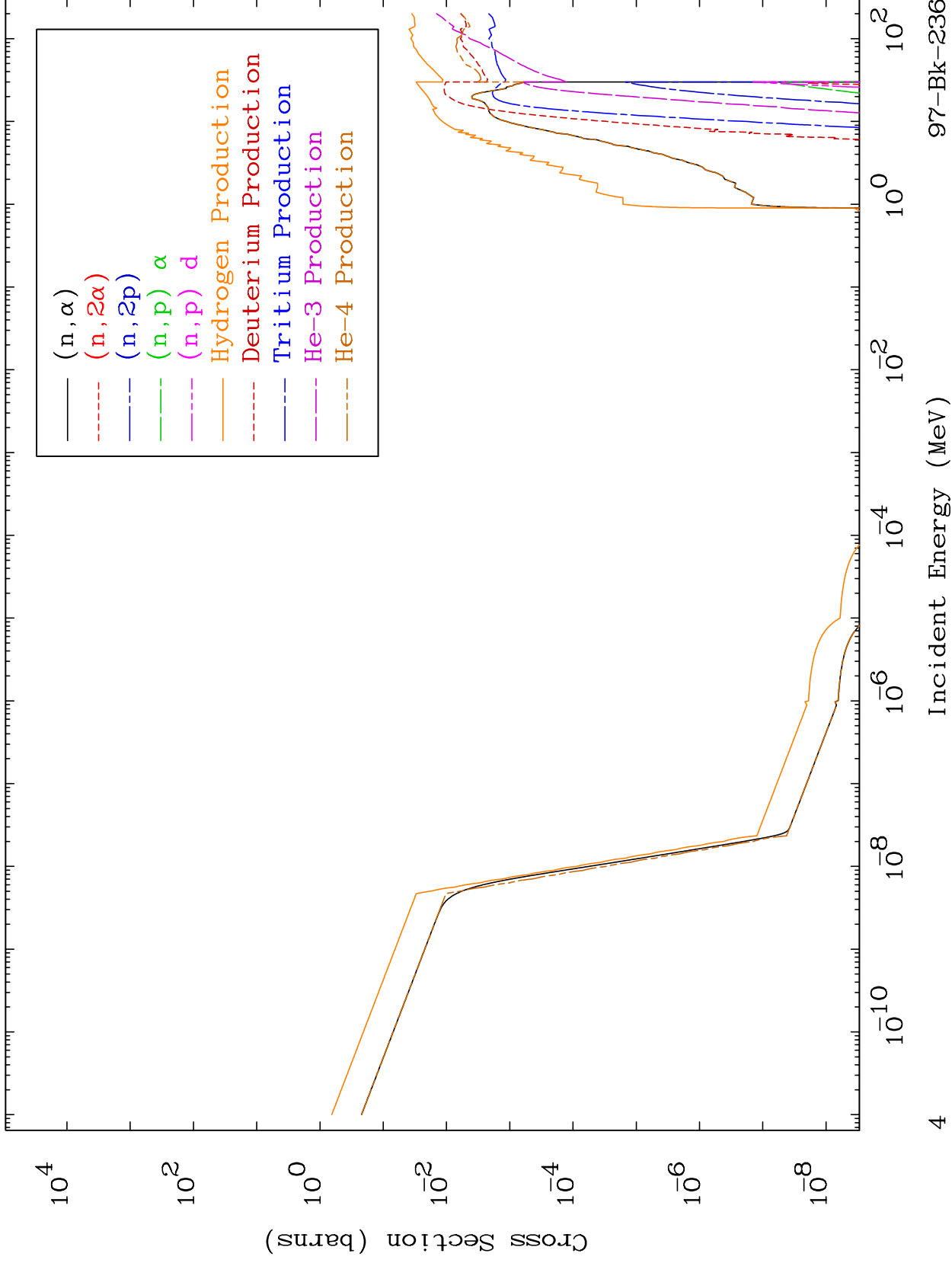


MAT 9713

Neutron Absorption
293 Kelvin Cross Sections

97-Bk-236

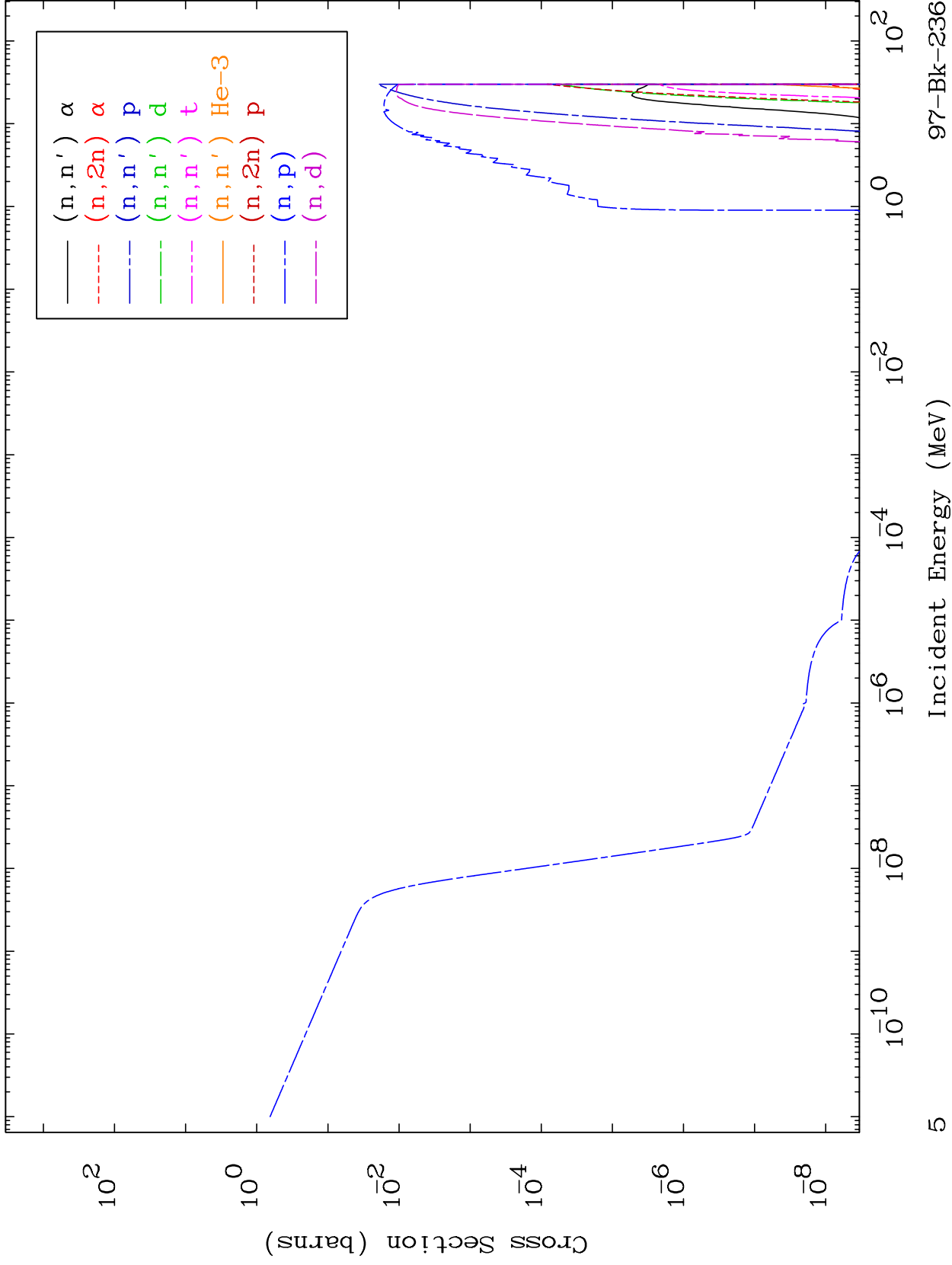




MAT 9713

Charged Particle
293 Kelvin Cross Sections

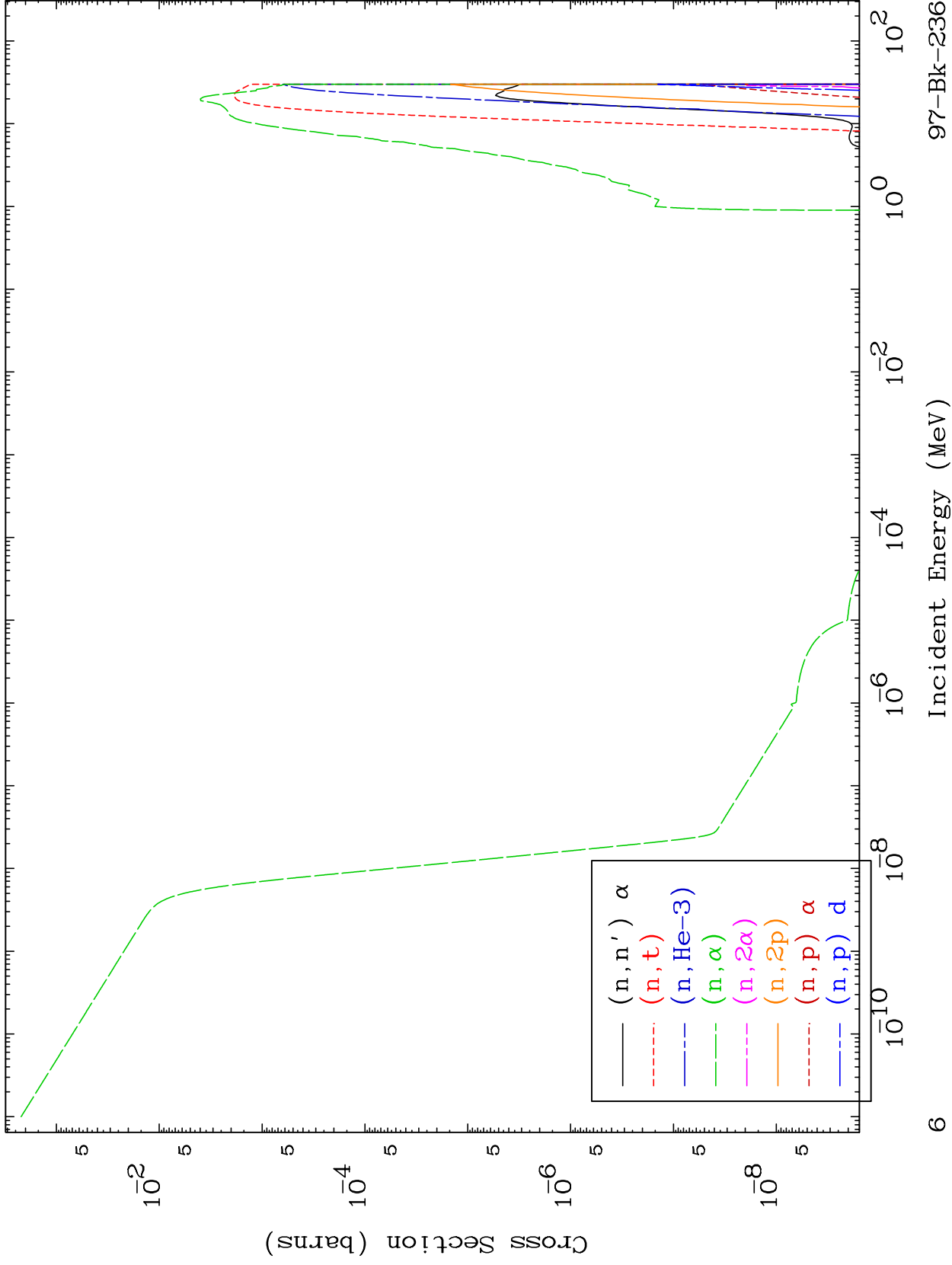
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Charged Particle
293 Kelvin Cross Sections

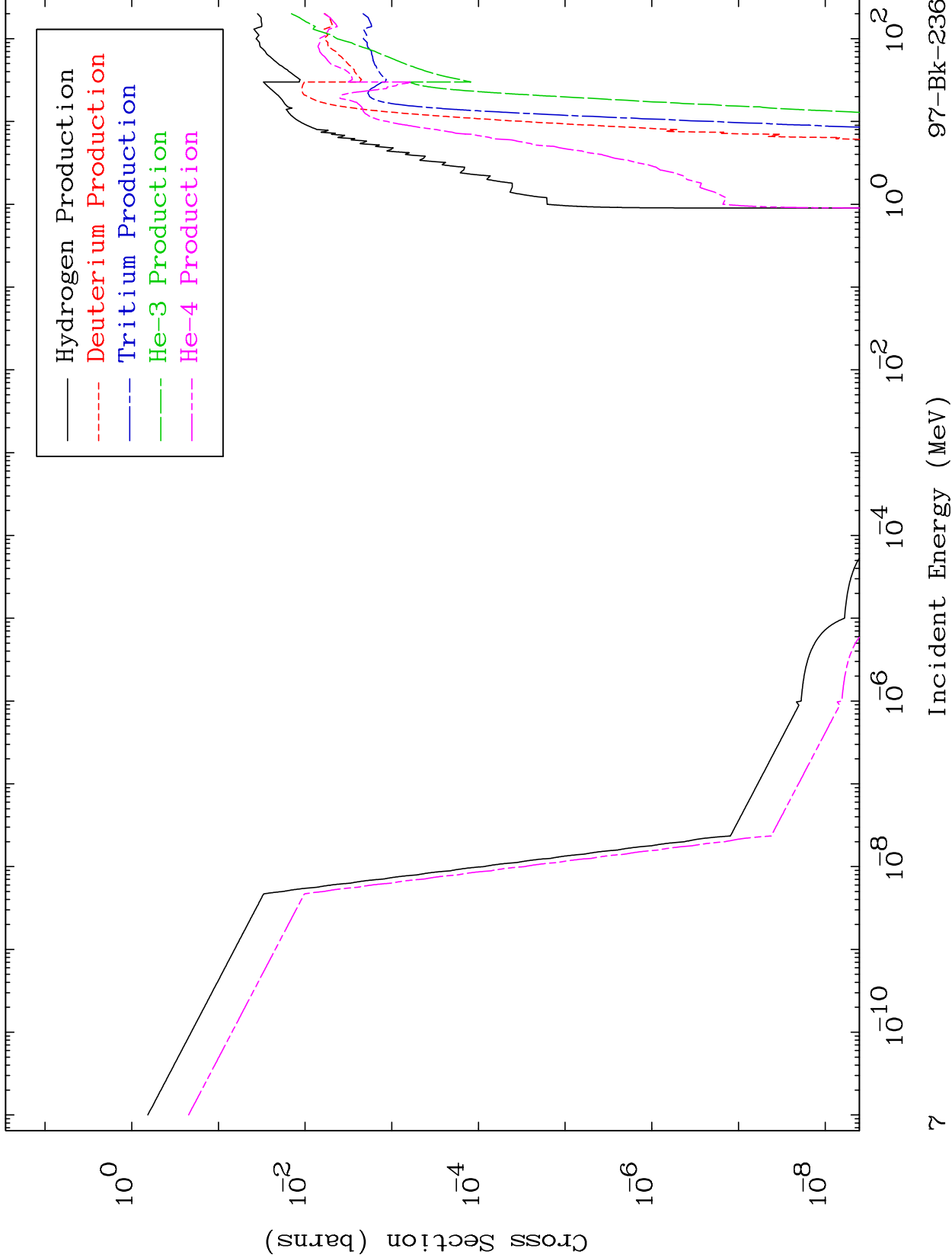
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MAT 9713

Particle Production
293 Kelvin Cross Sections

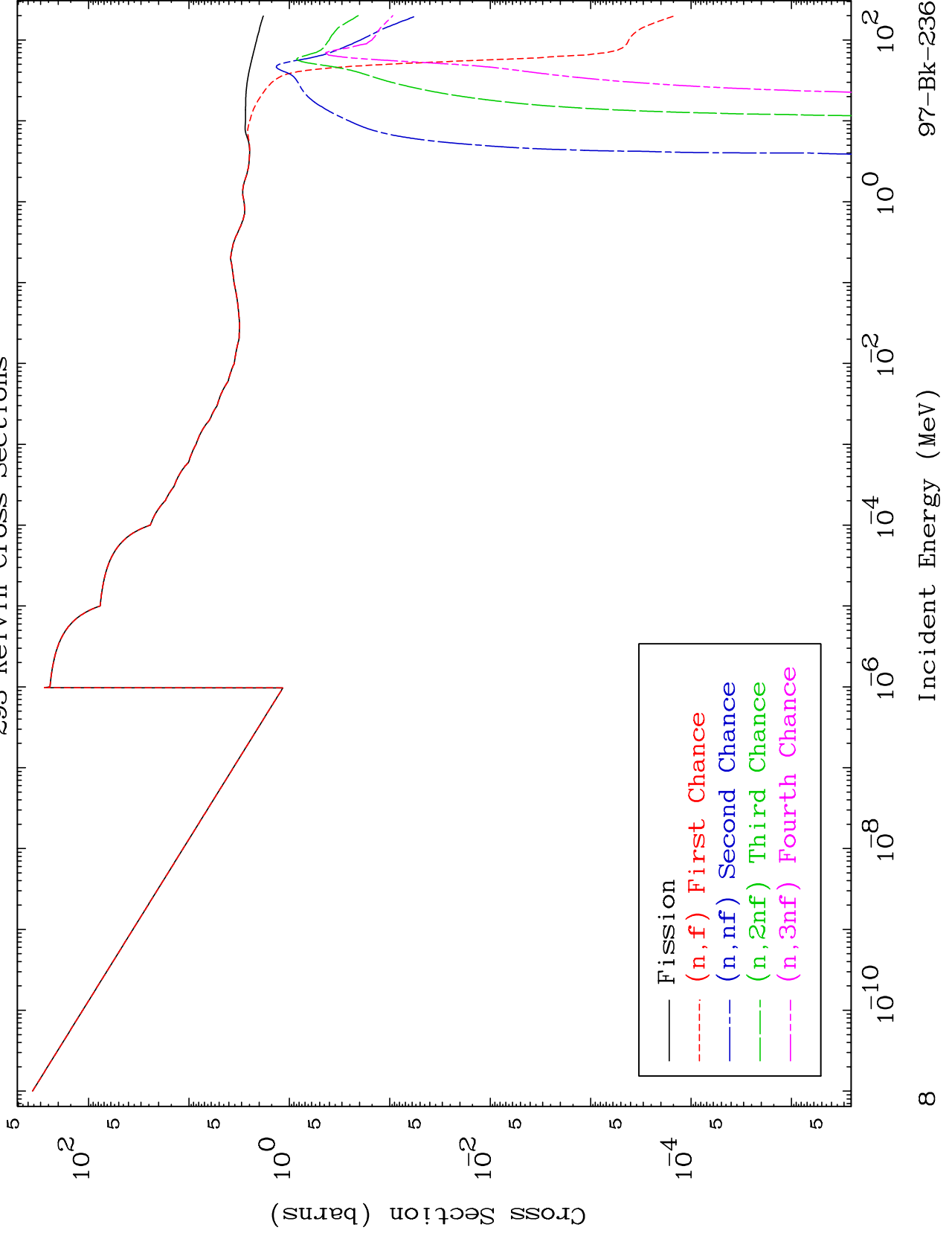
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MAT 9713

Fission
293 Kelvin Cross Sections

97-Bk-236

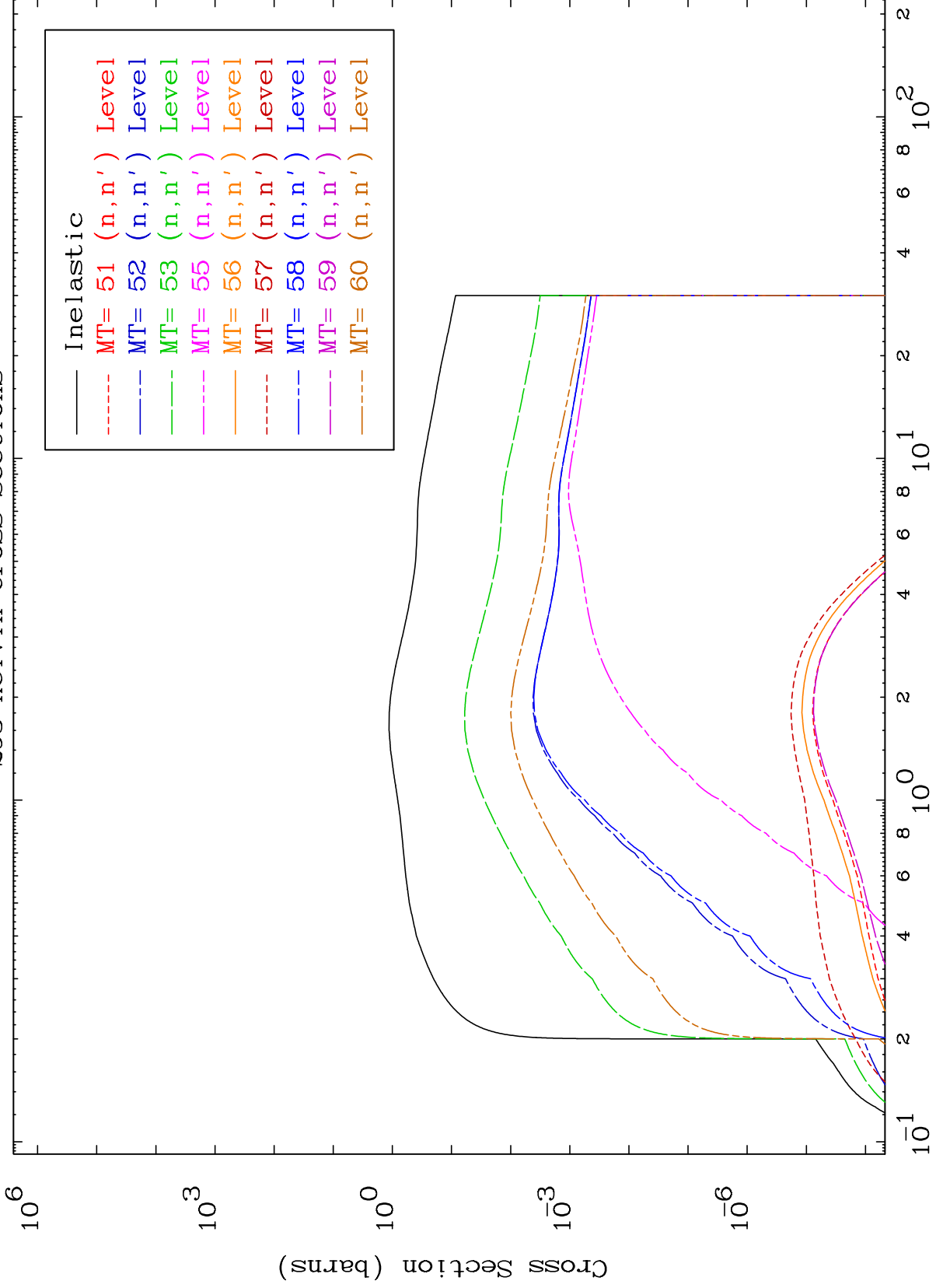


MAT 9713

(n,n') Levels

97-Bk-236

293 Kelvin Cross Sections



Incident Energy (MeV)

9

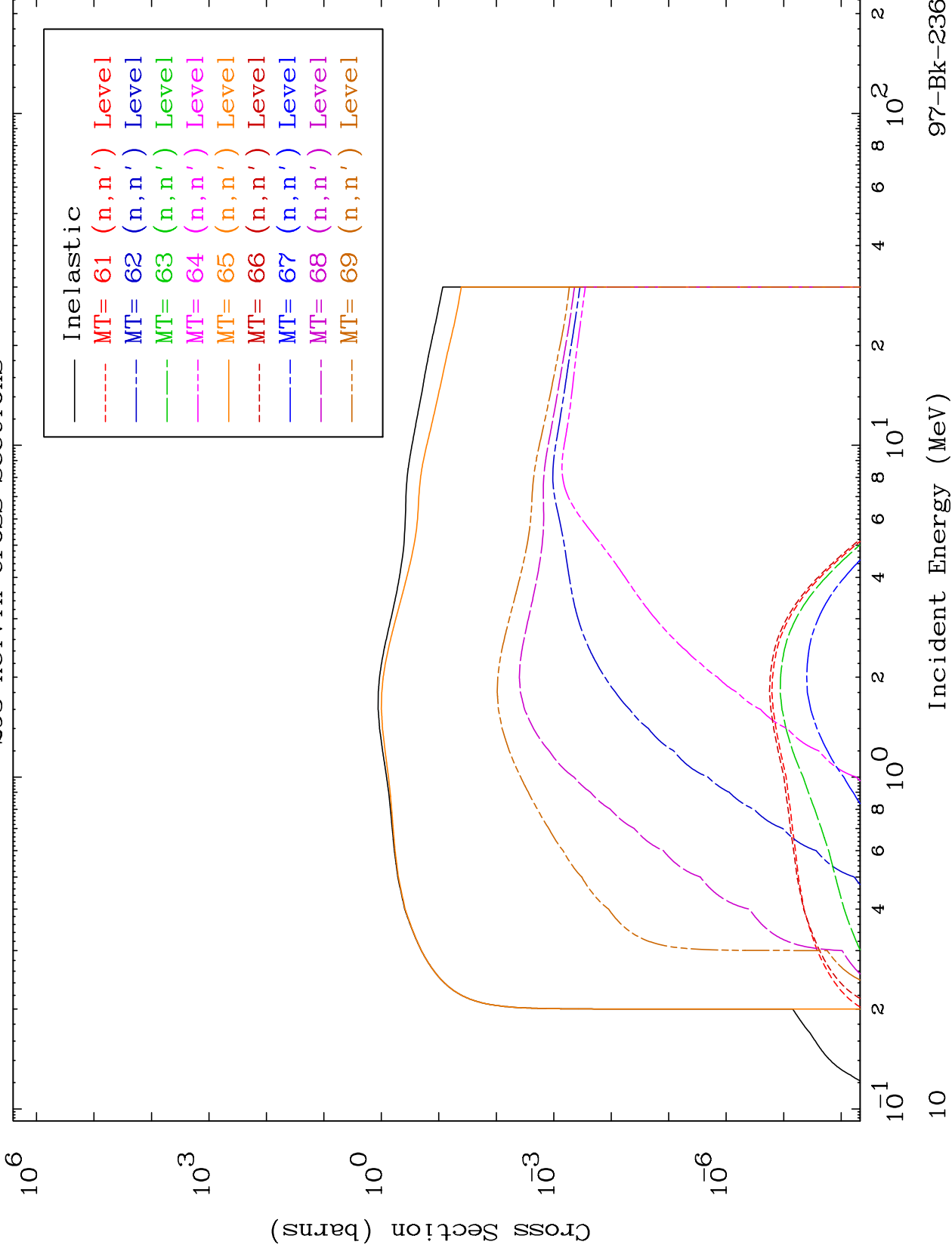
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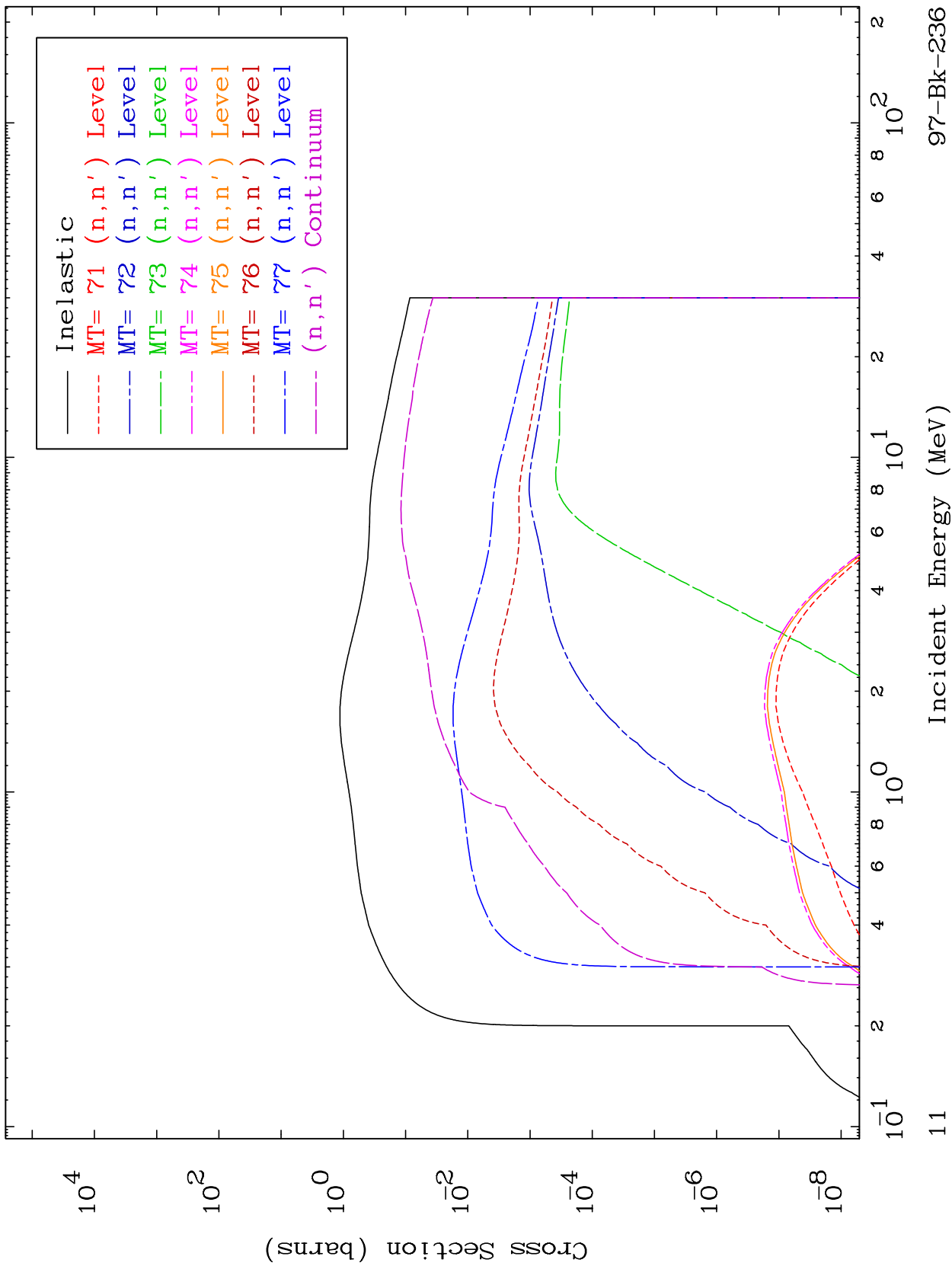
MAT 9713

(n,n') Levels

97-Bk-236

293 Kelvin Cross Sections

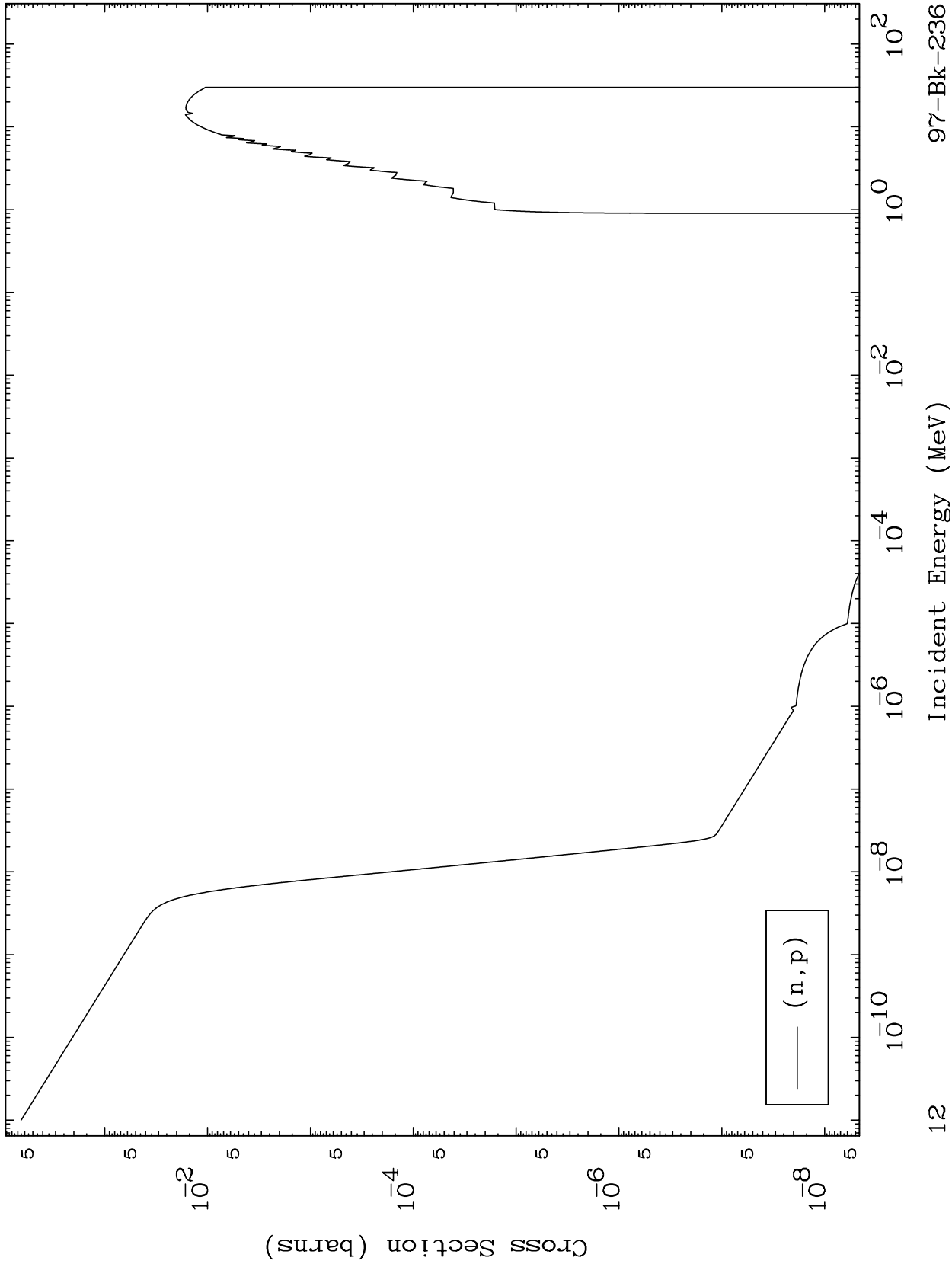




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(n,p) Levels
293 Kelvin Cross Sections

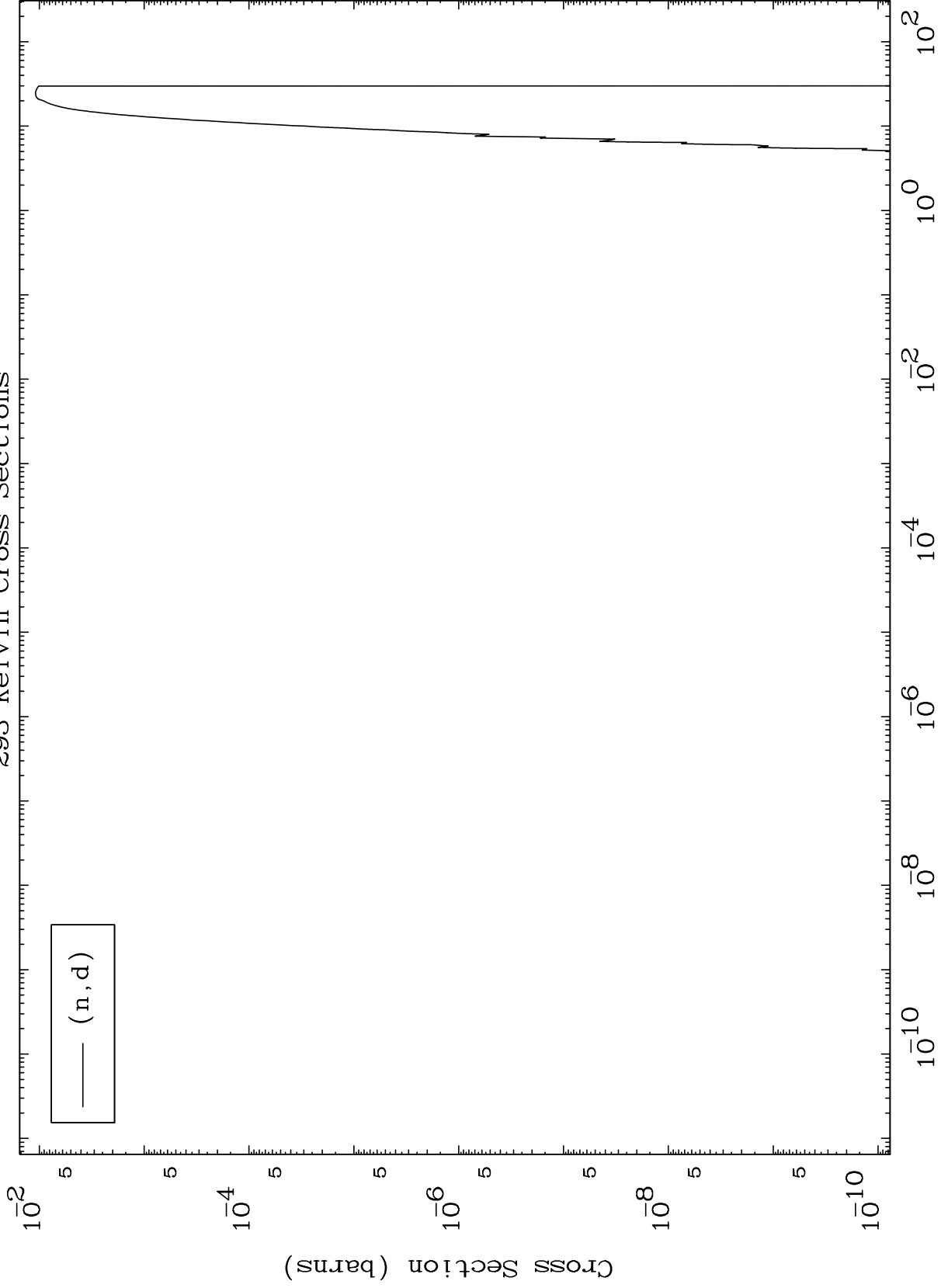
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MAT 9713

(n,d) Levels
293 Kelvin Cross Sections

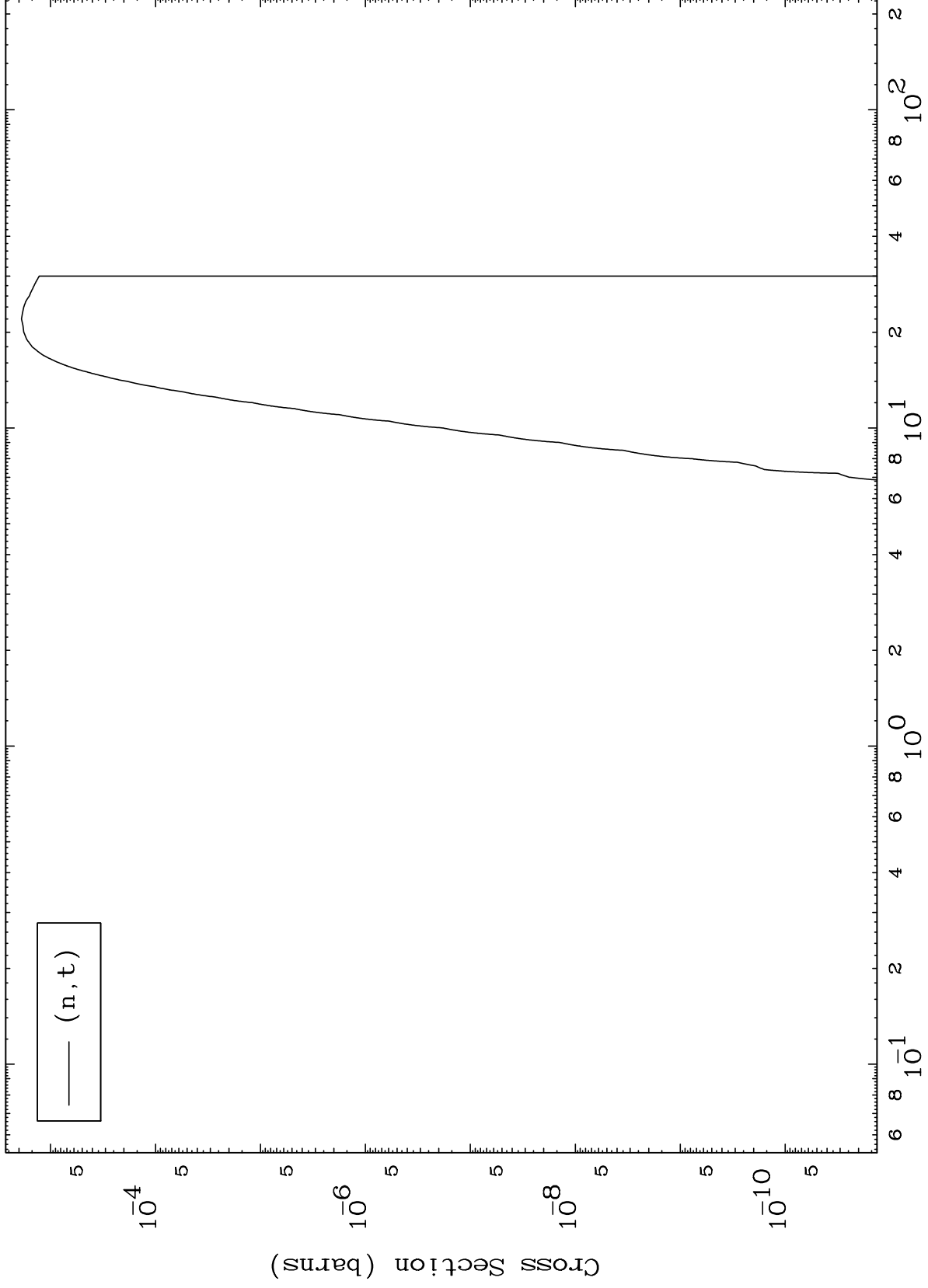
97-Bk-236



MAT 9713

293 (n,t) Levels
Kelvin Cross Sections

97-Bk-236



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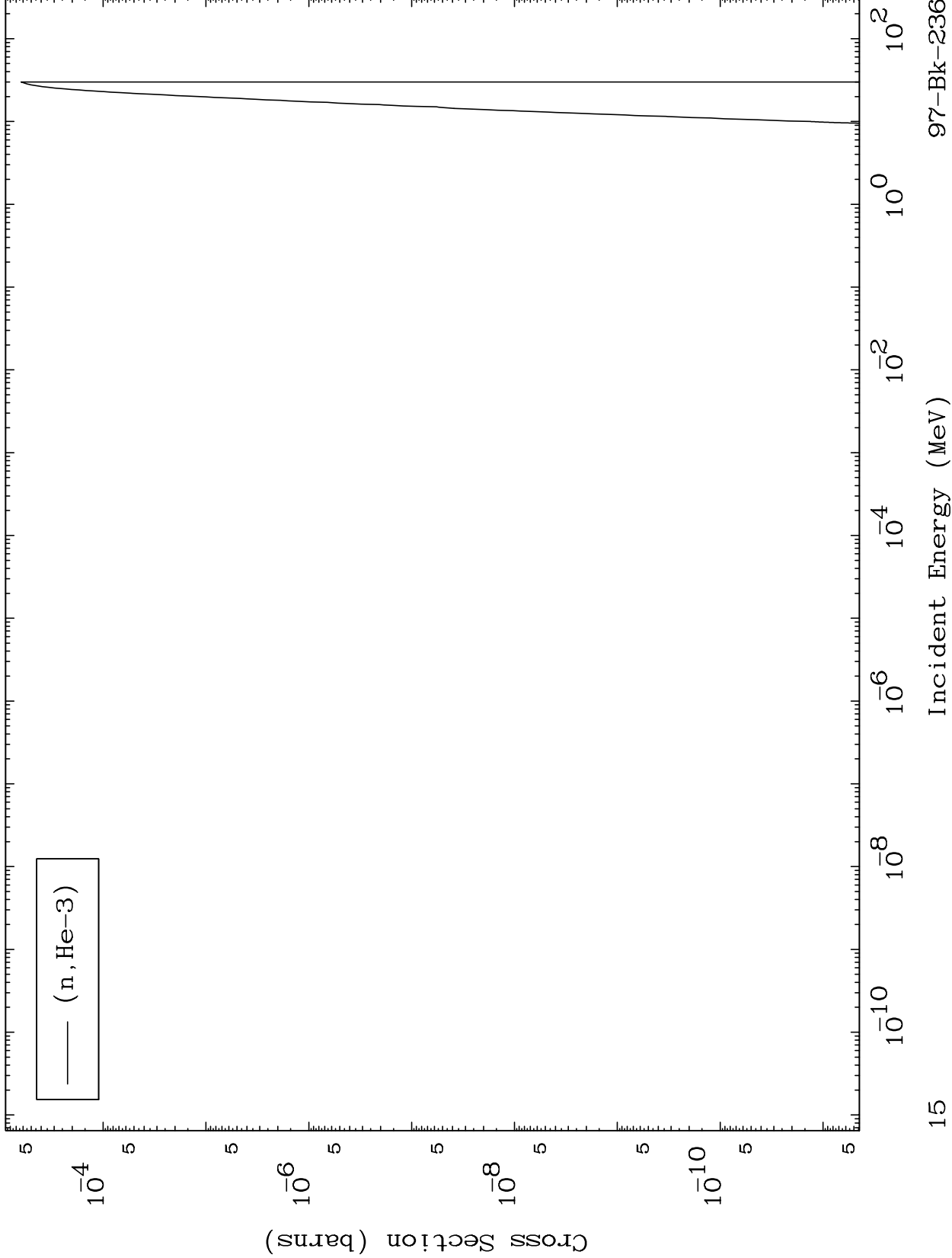
Incident Energy (MeV)

97-Bk-236

MAT 9713

(n,He3) Levels
293 Kelvin Cross Sections

97-Bk-236



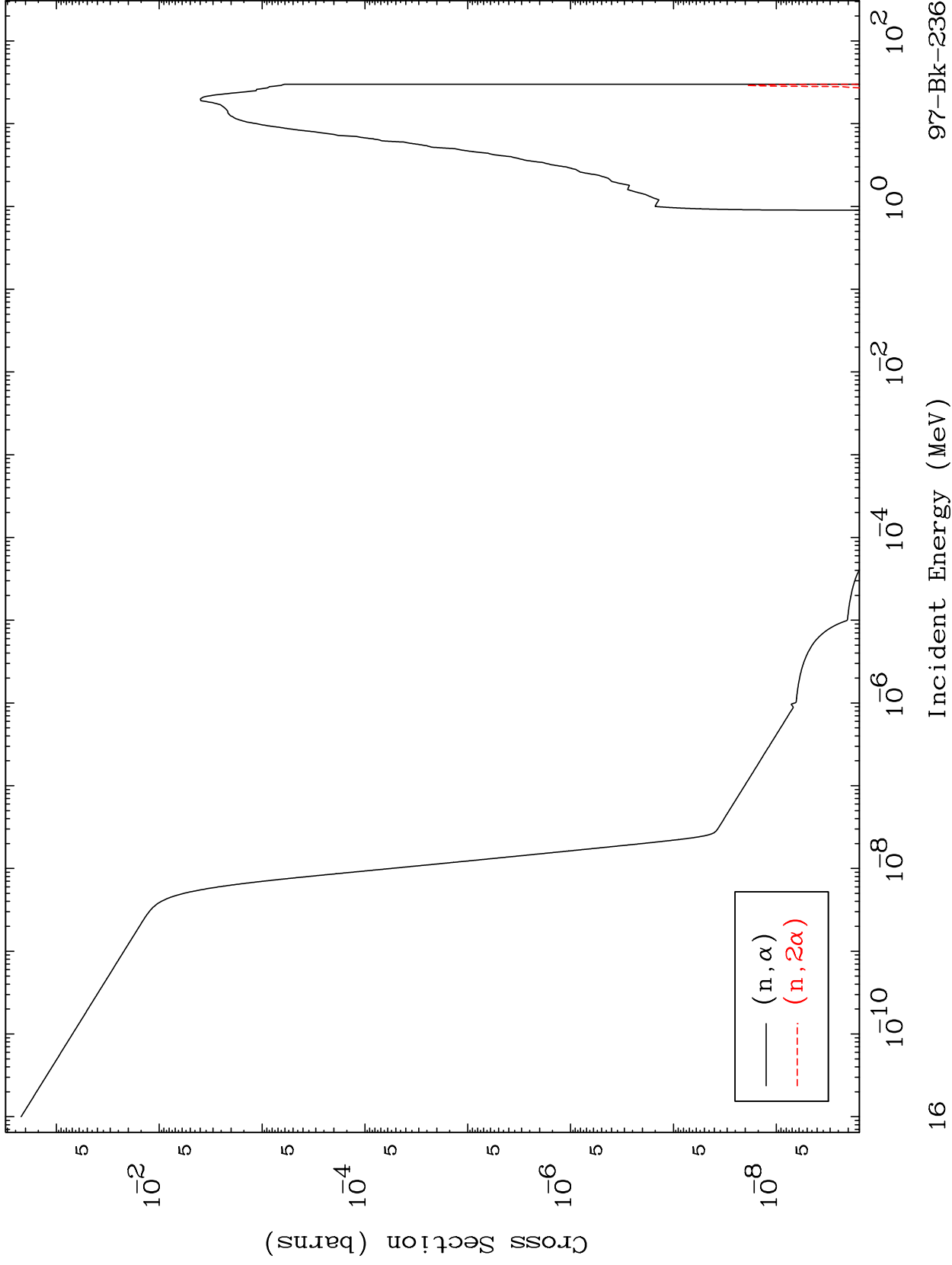
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97-Bk-236

MAT 9713

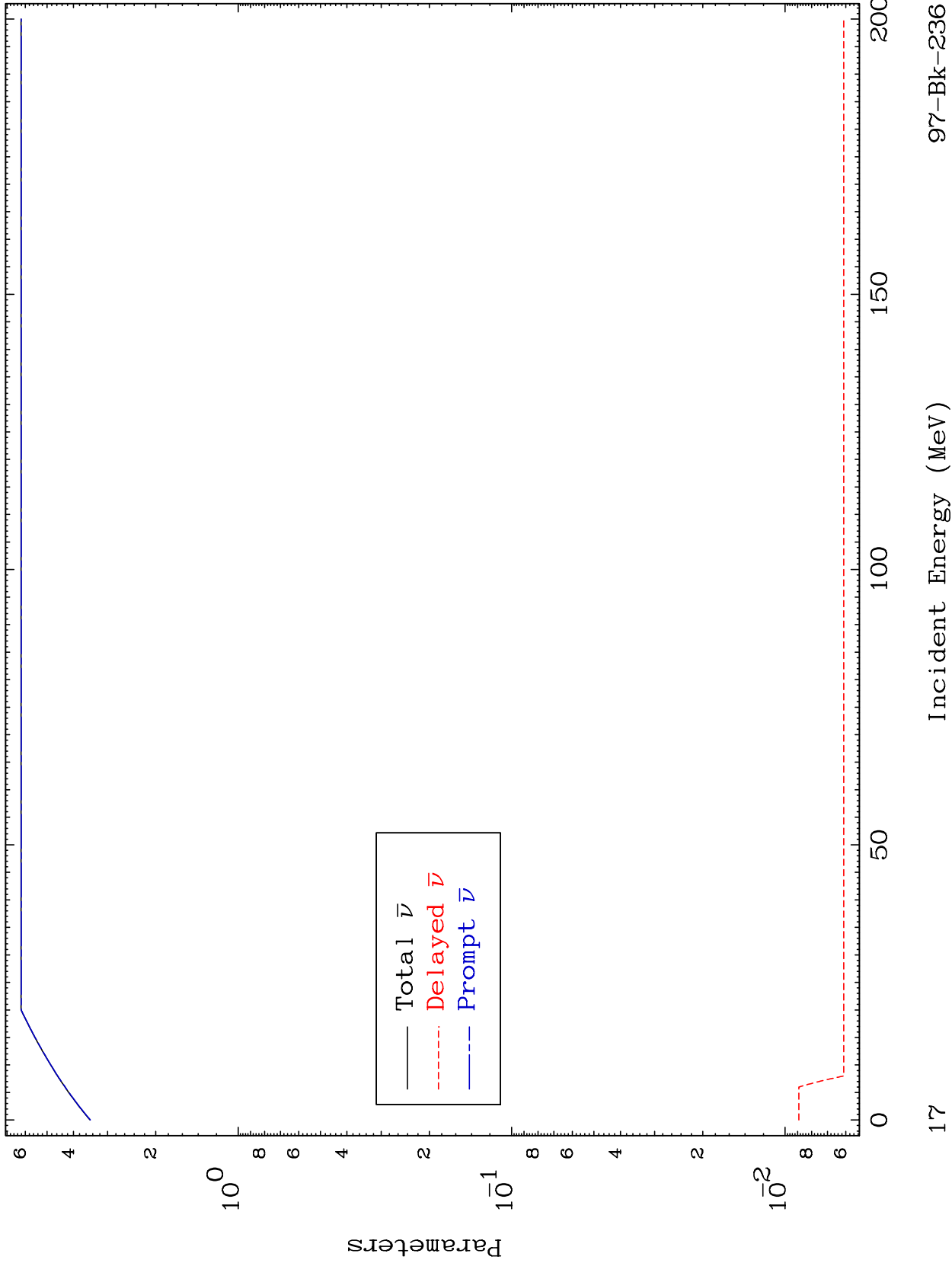
(n, α) Levels
293 Kelvin Cross Sections

97-Bk-236



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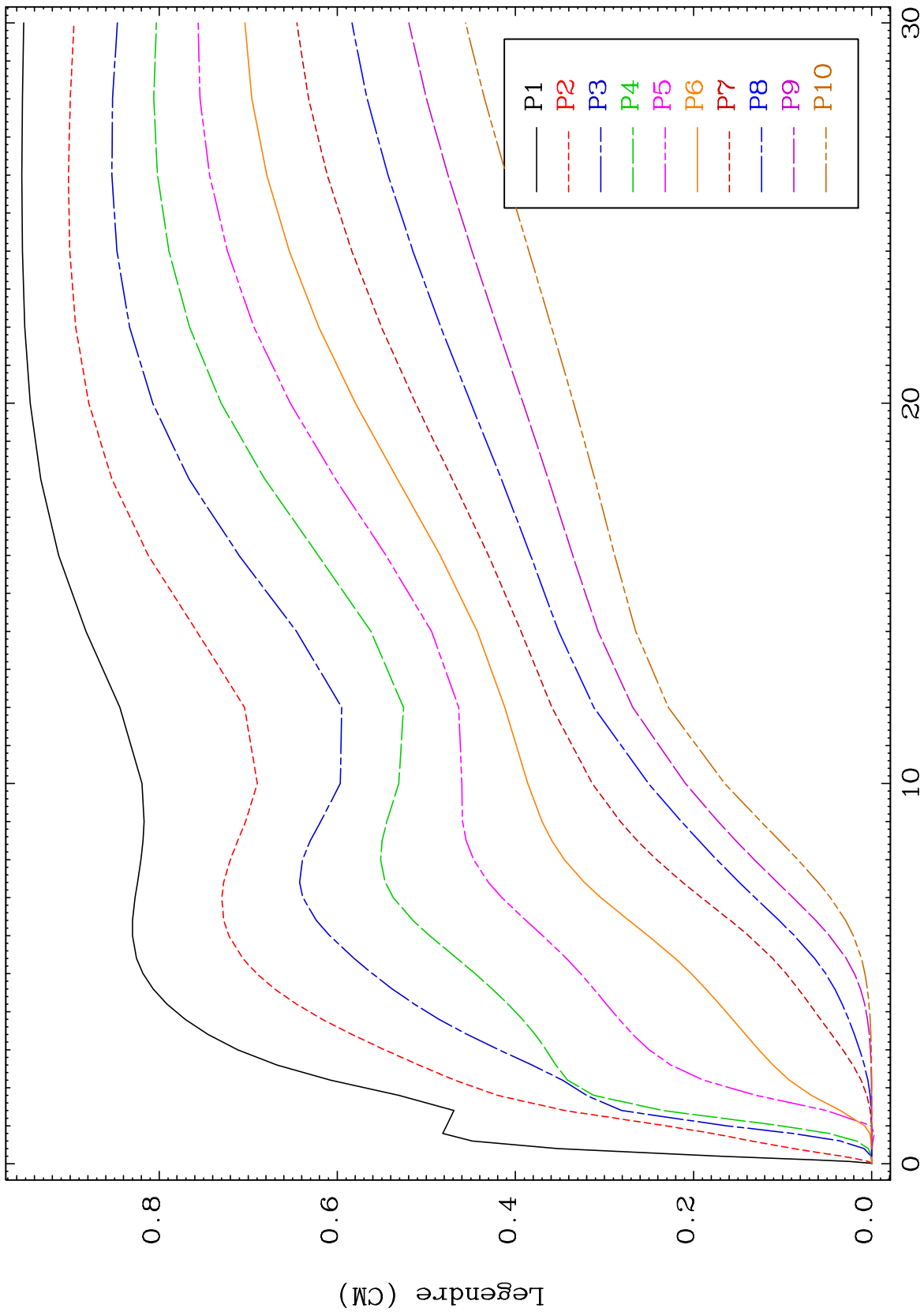
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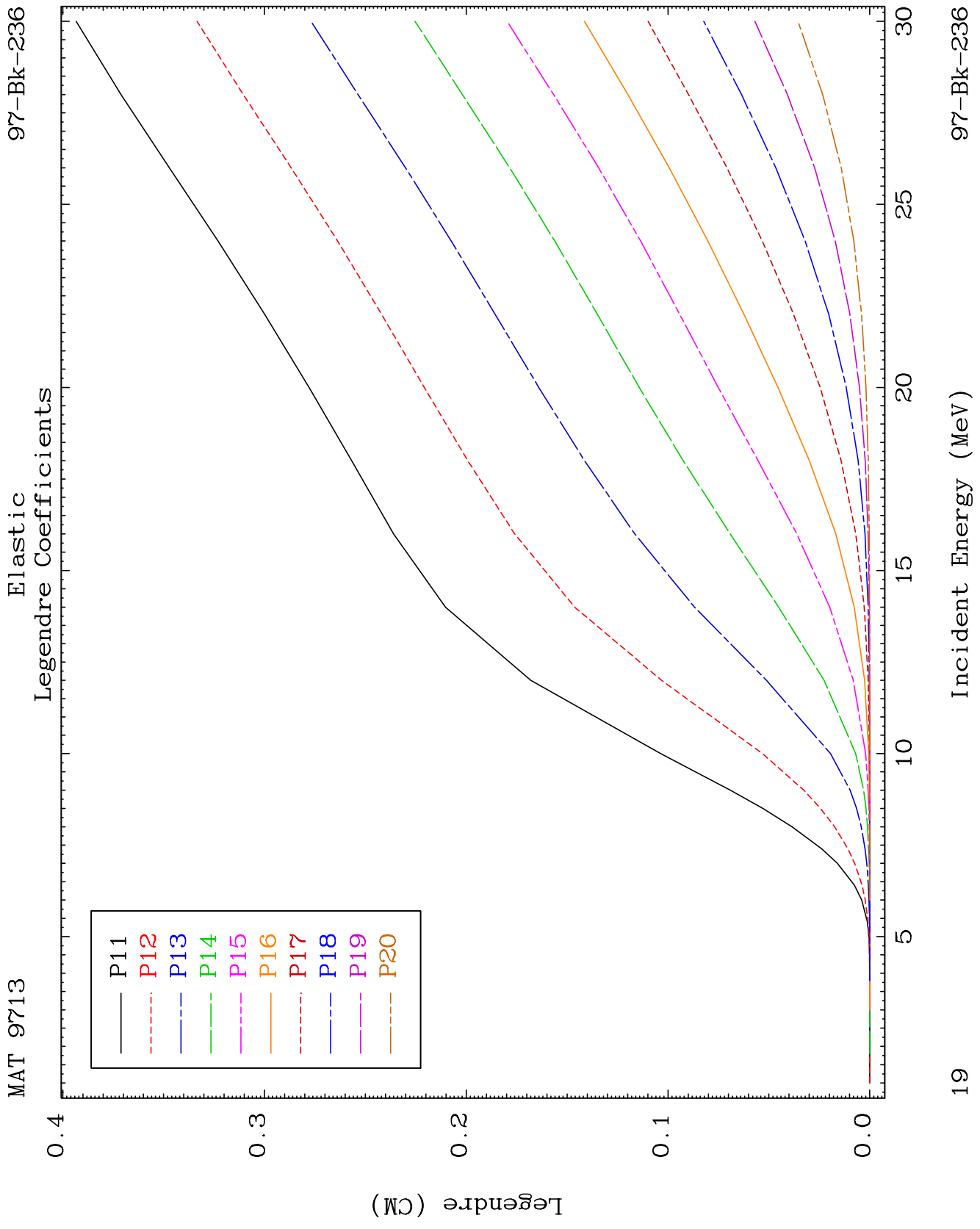
Elastic Legendre Coefficients

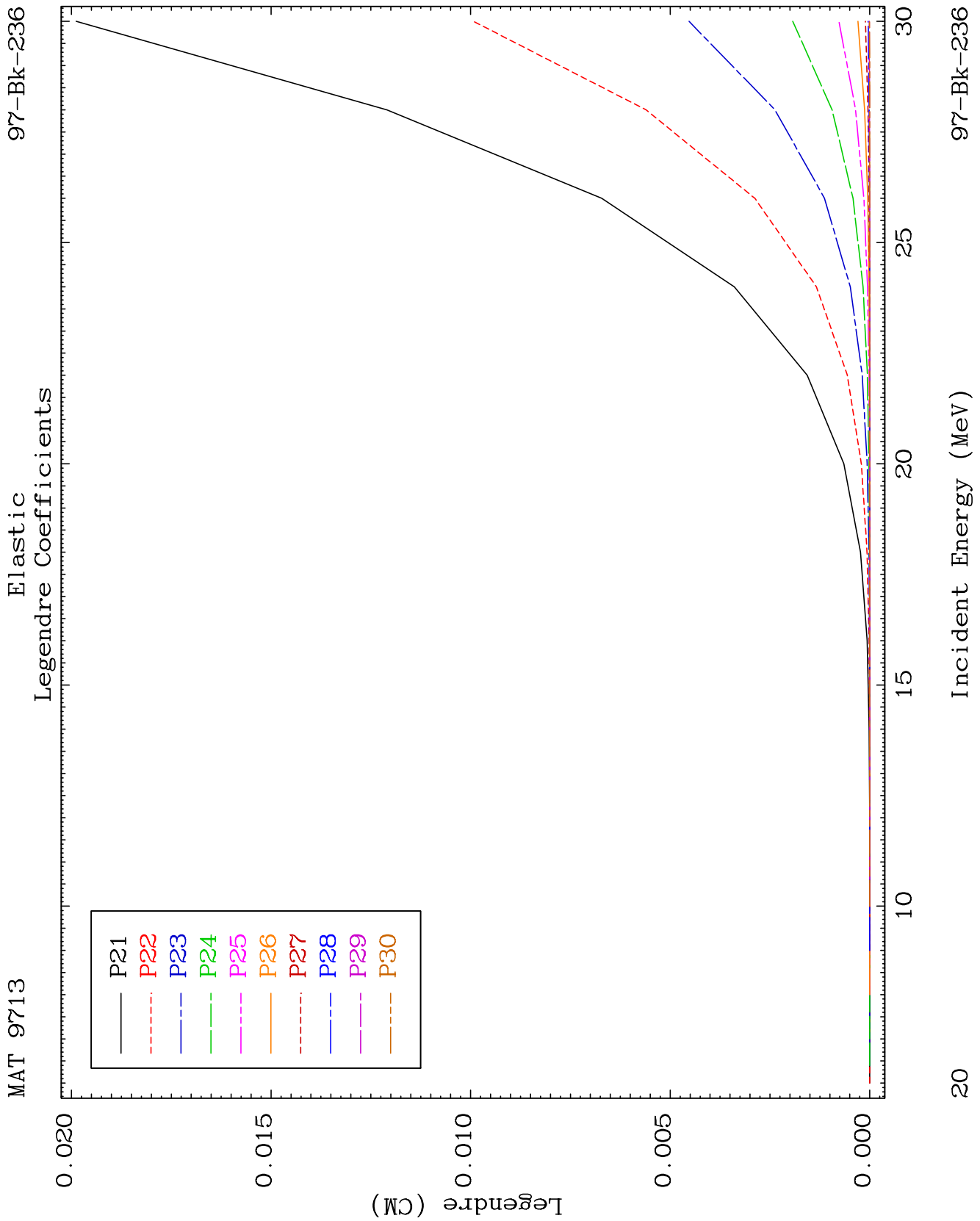


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Incident Energy (MeV)

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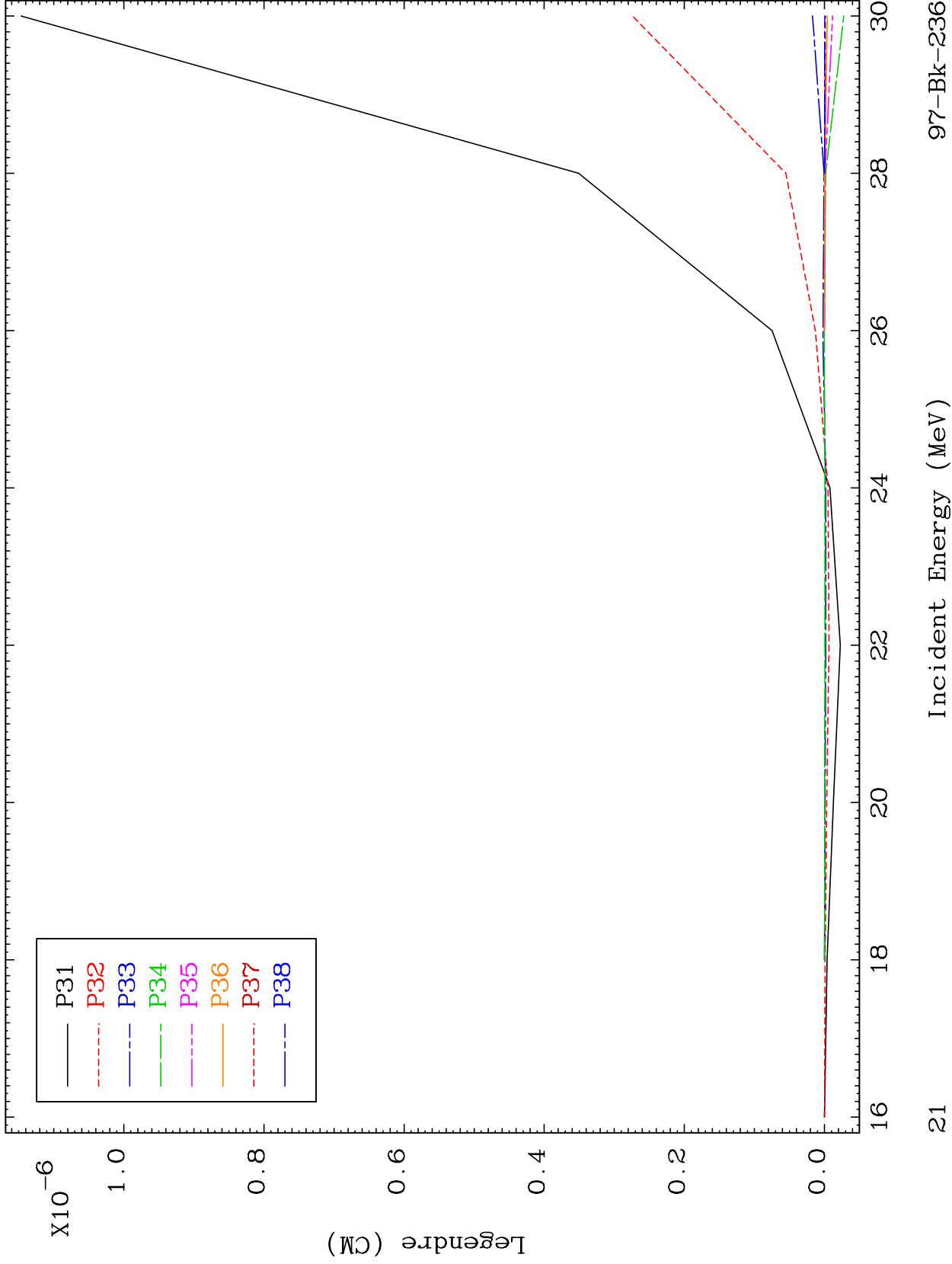




MAT 9713

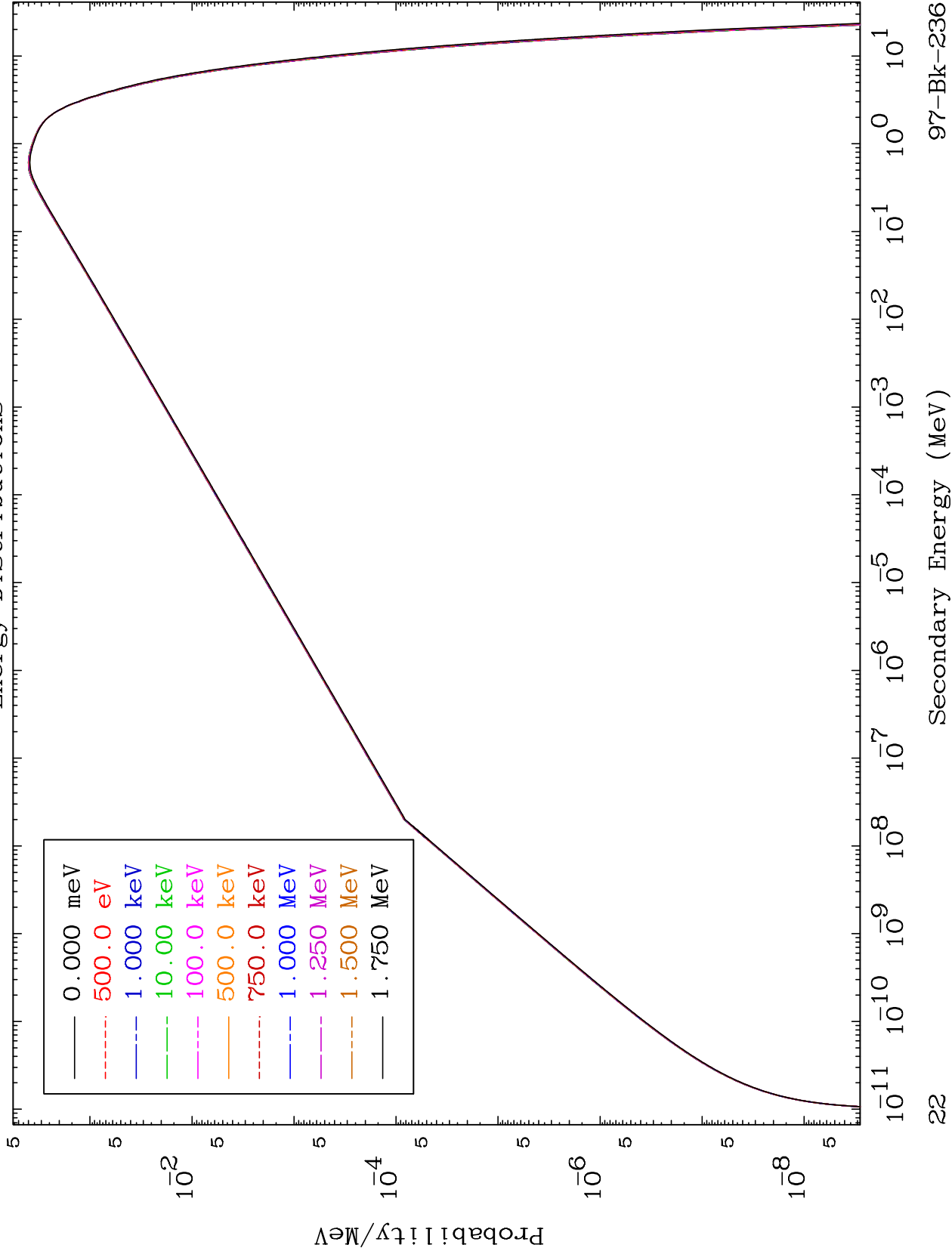
Elastic Legendre Coefficients

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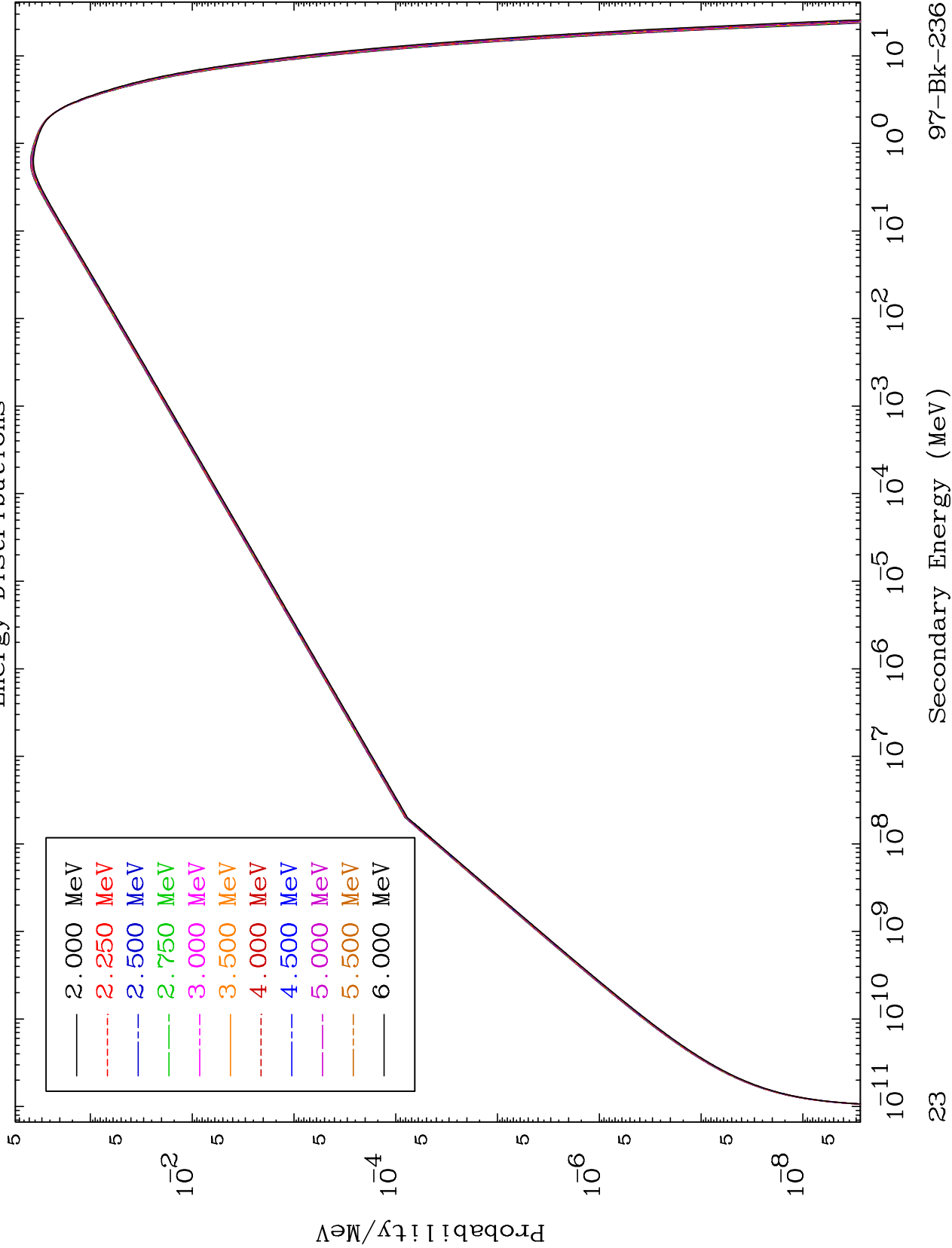


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Fission Energy Distributions



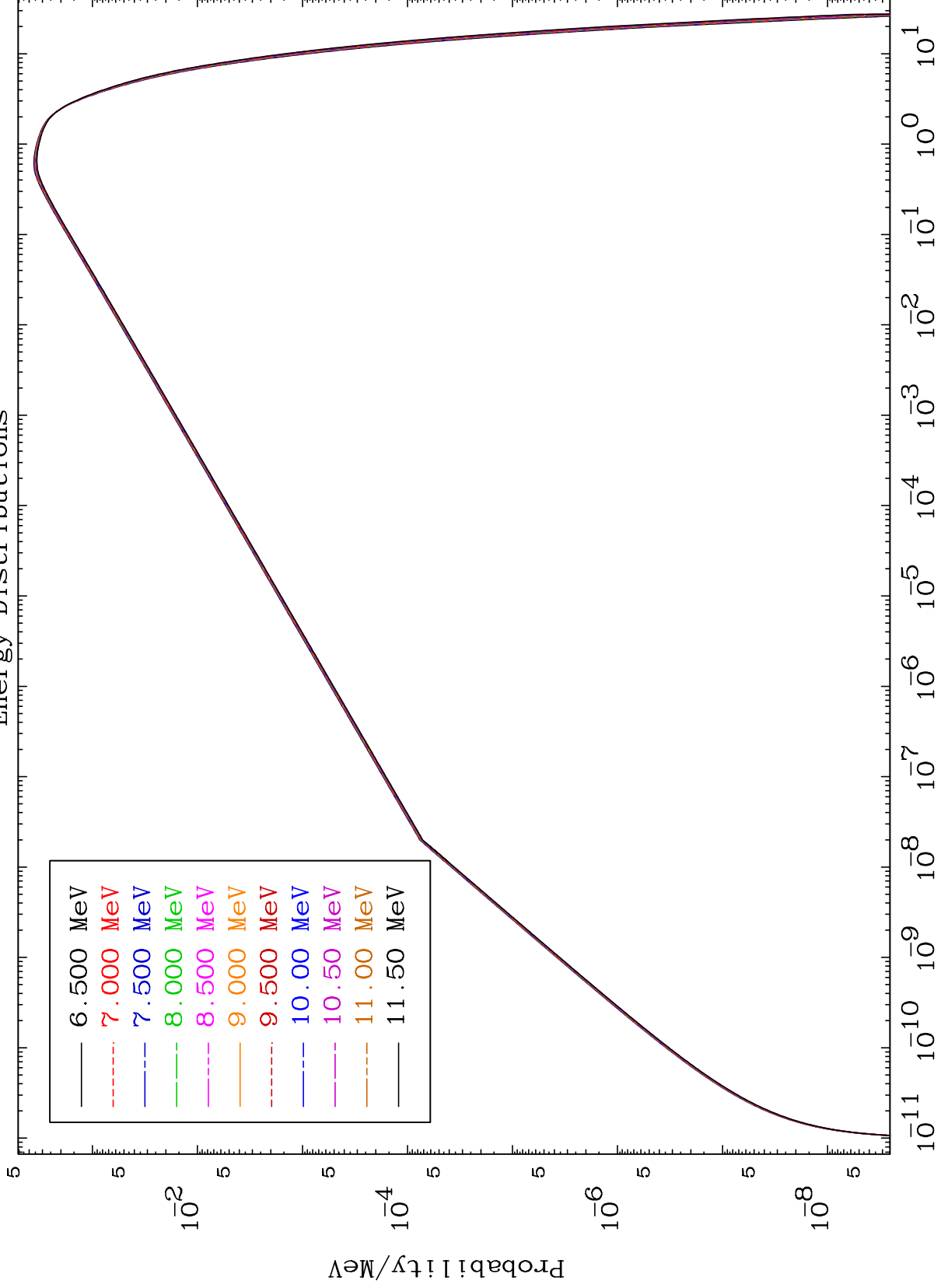
Fission Energy Distributions



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Fission Energy Distributions

97-Bk-236

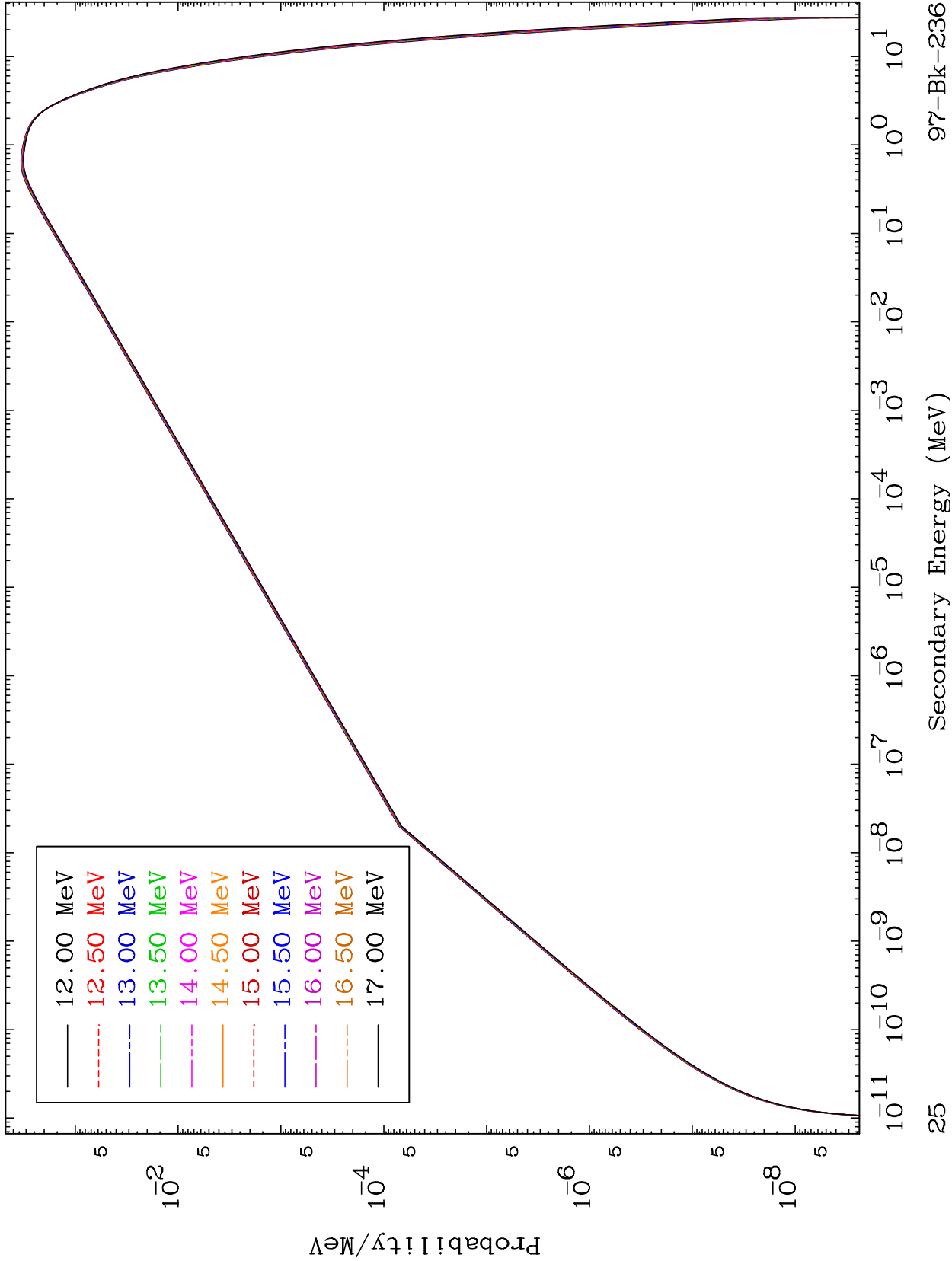


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Secondary Energy (MeV)

97-Bk-236

Fission Energy Distributions



Fission Energy Distributions

