

Program EVALPLOT
(Version 2021-1)

by

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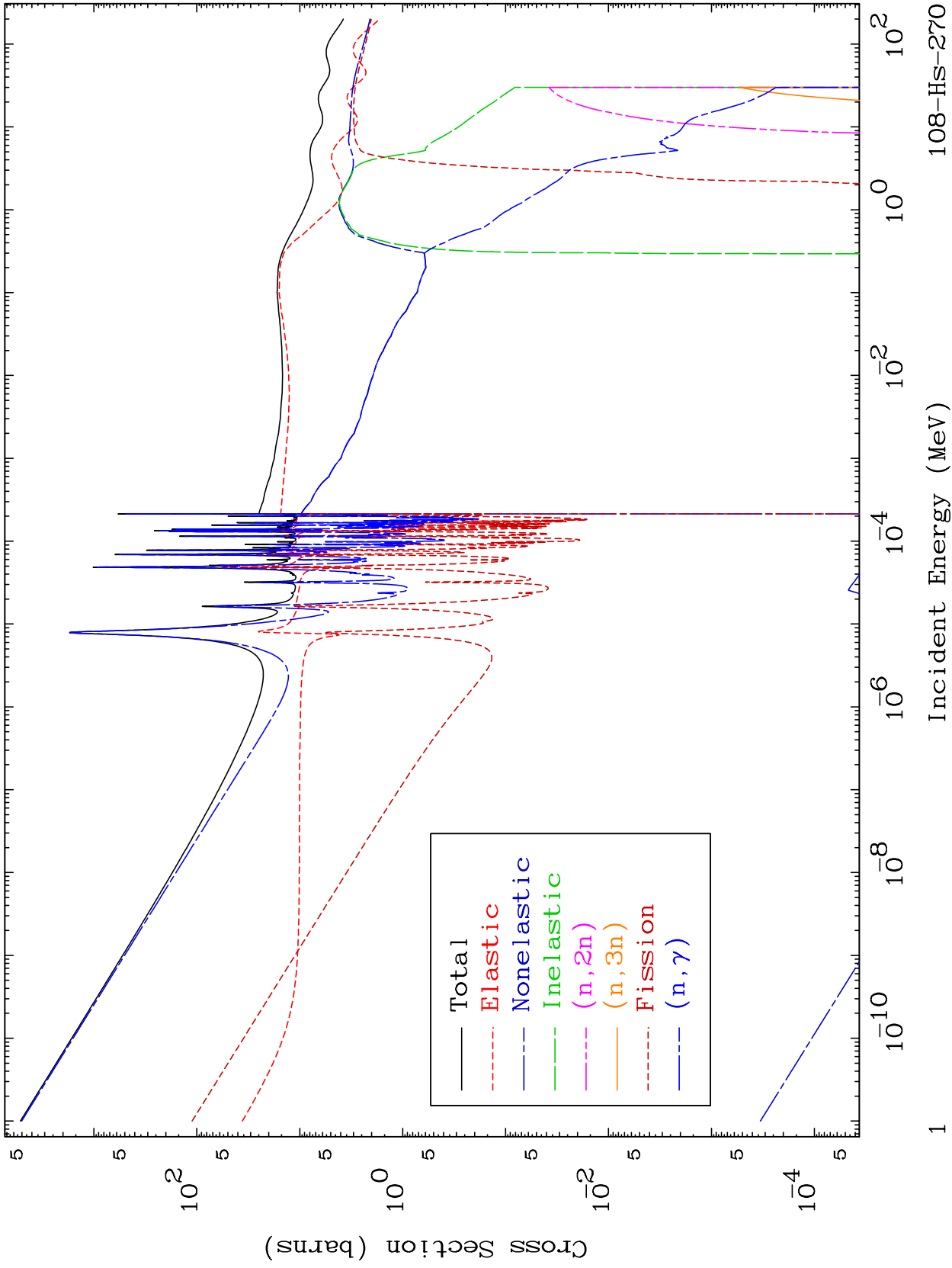
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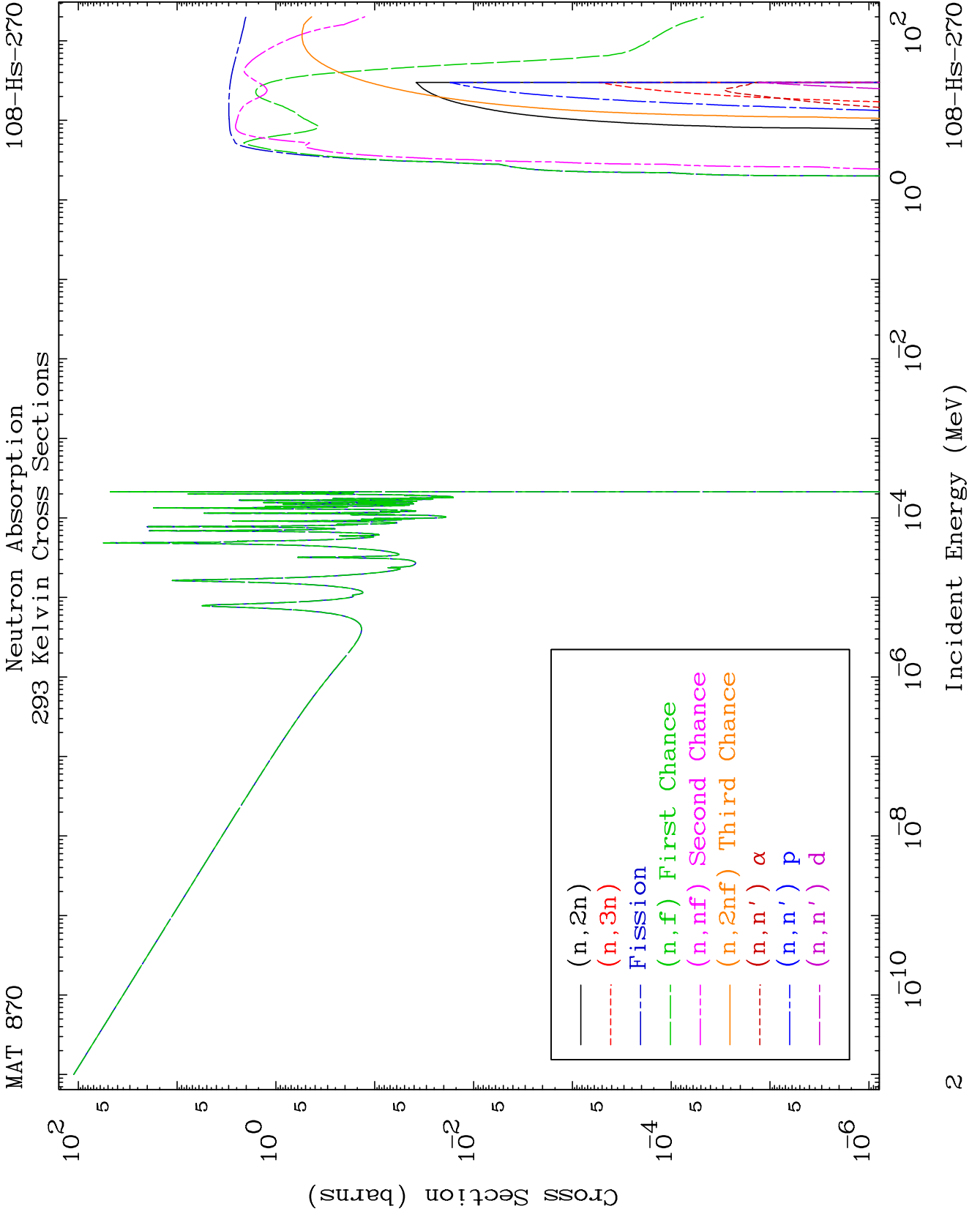
Press Mouse Button to Start

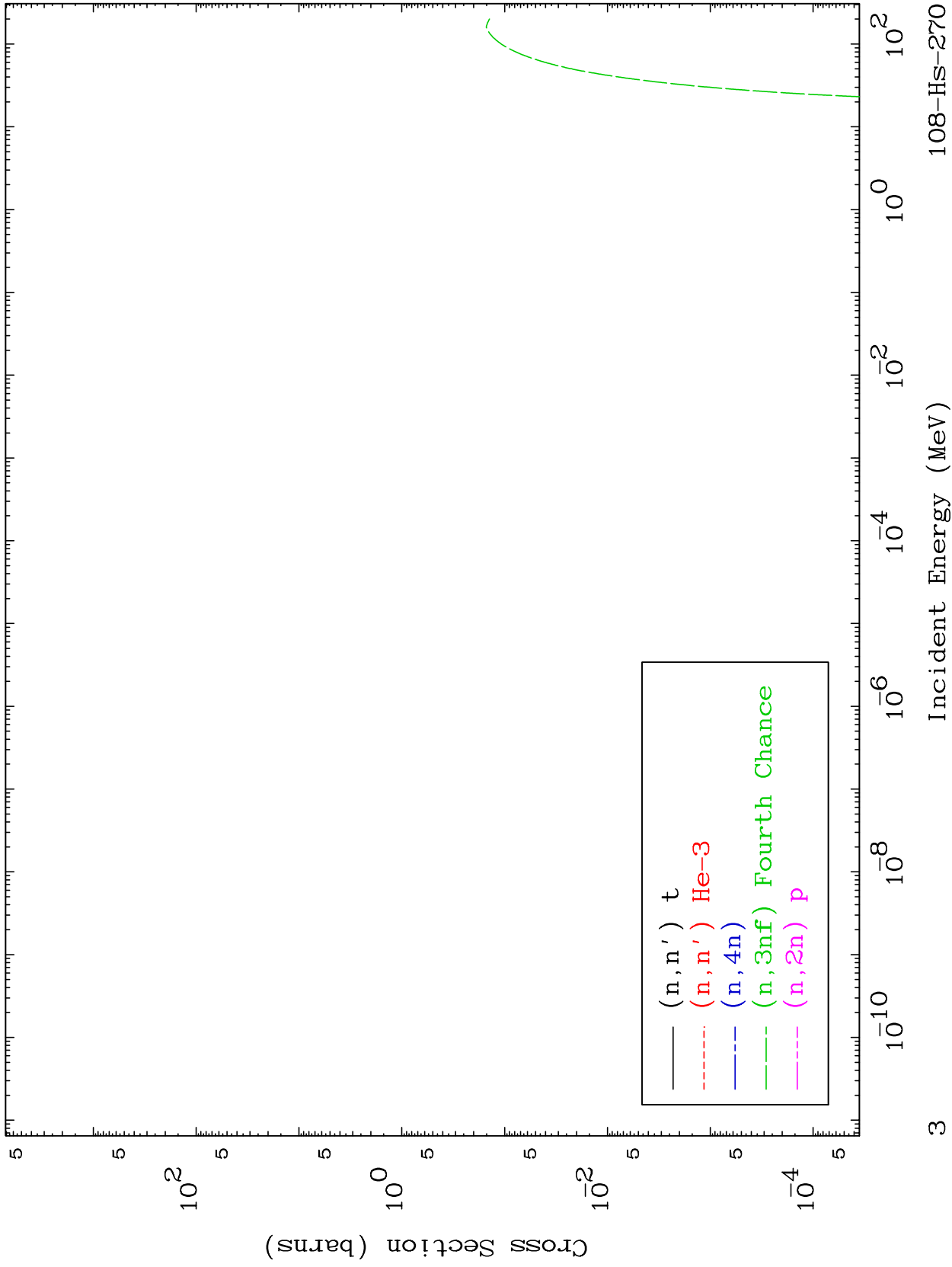
MAT 870

Neutron Major
293 Kelvin Cross Sections

108-Hs-270



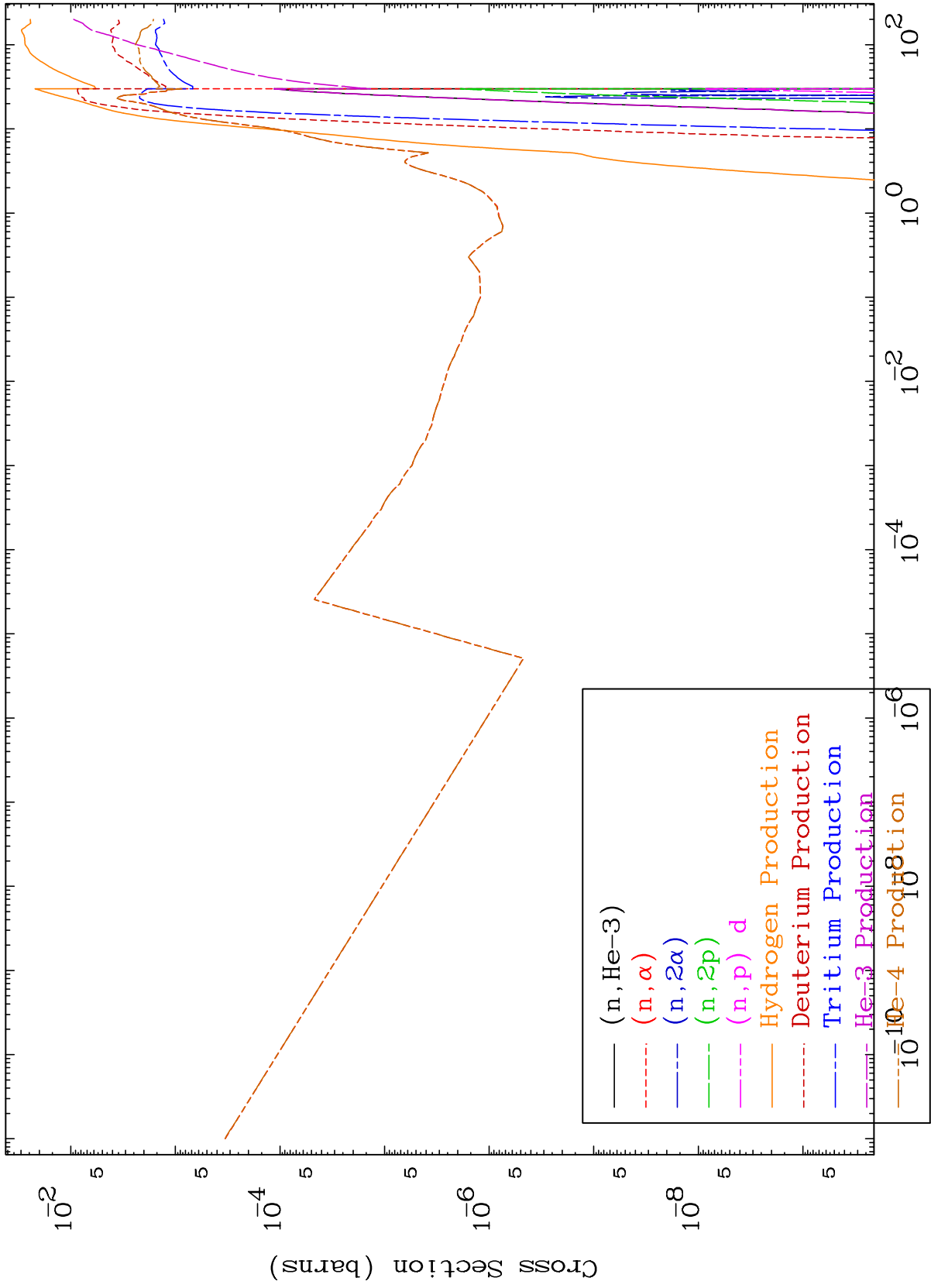




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Neutron Absorption
293 Kelvin Cross Sections

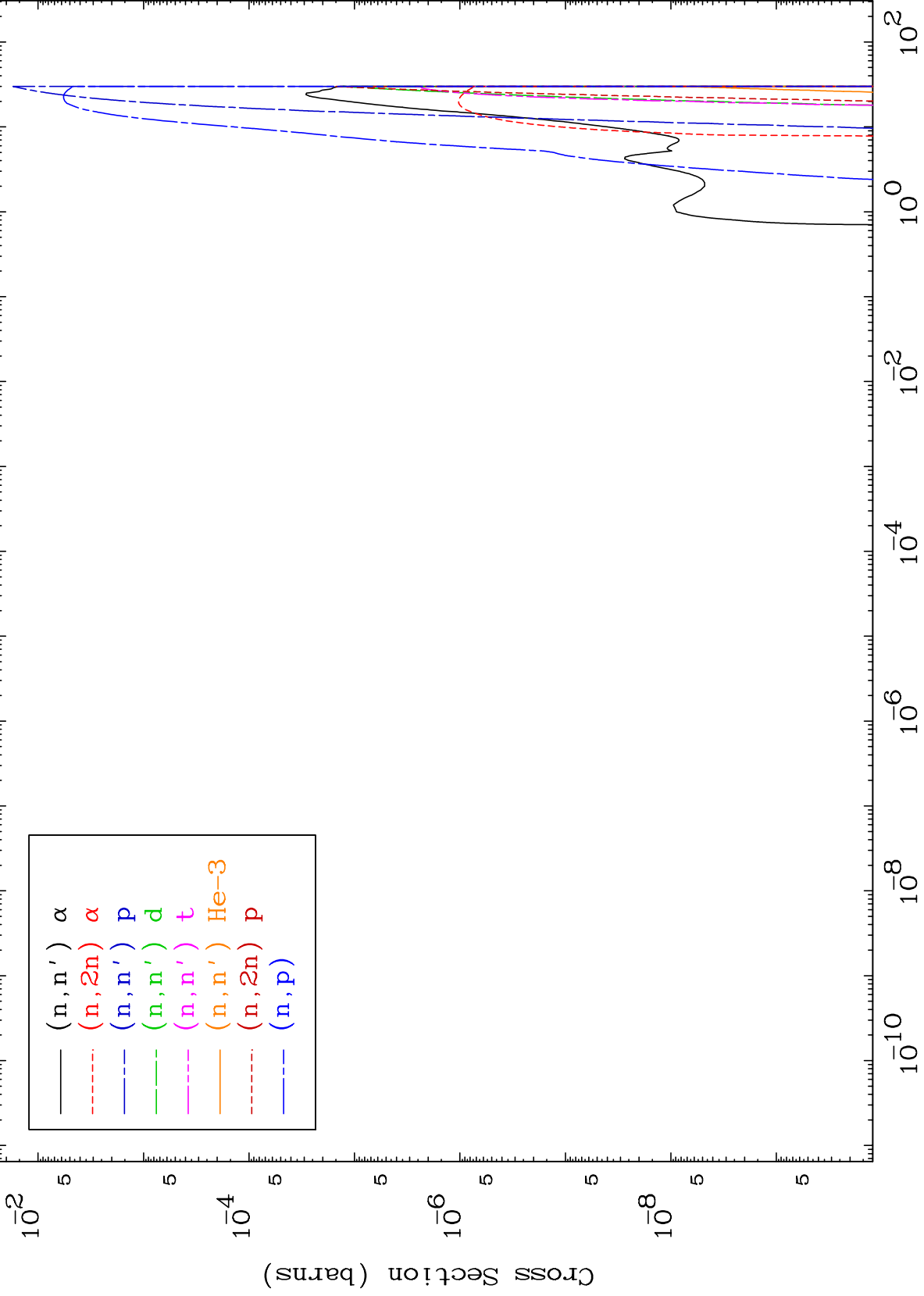
108-Hs-270



MAT 870

Charged Particle
293 Kelvin Cross Sections

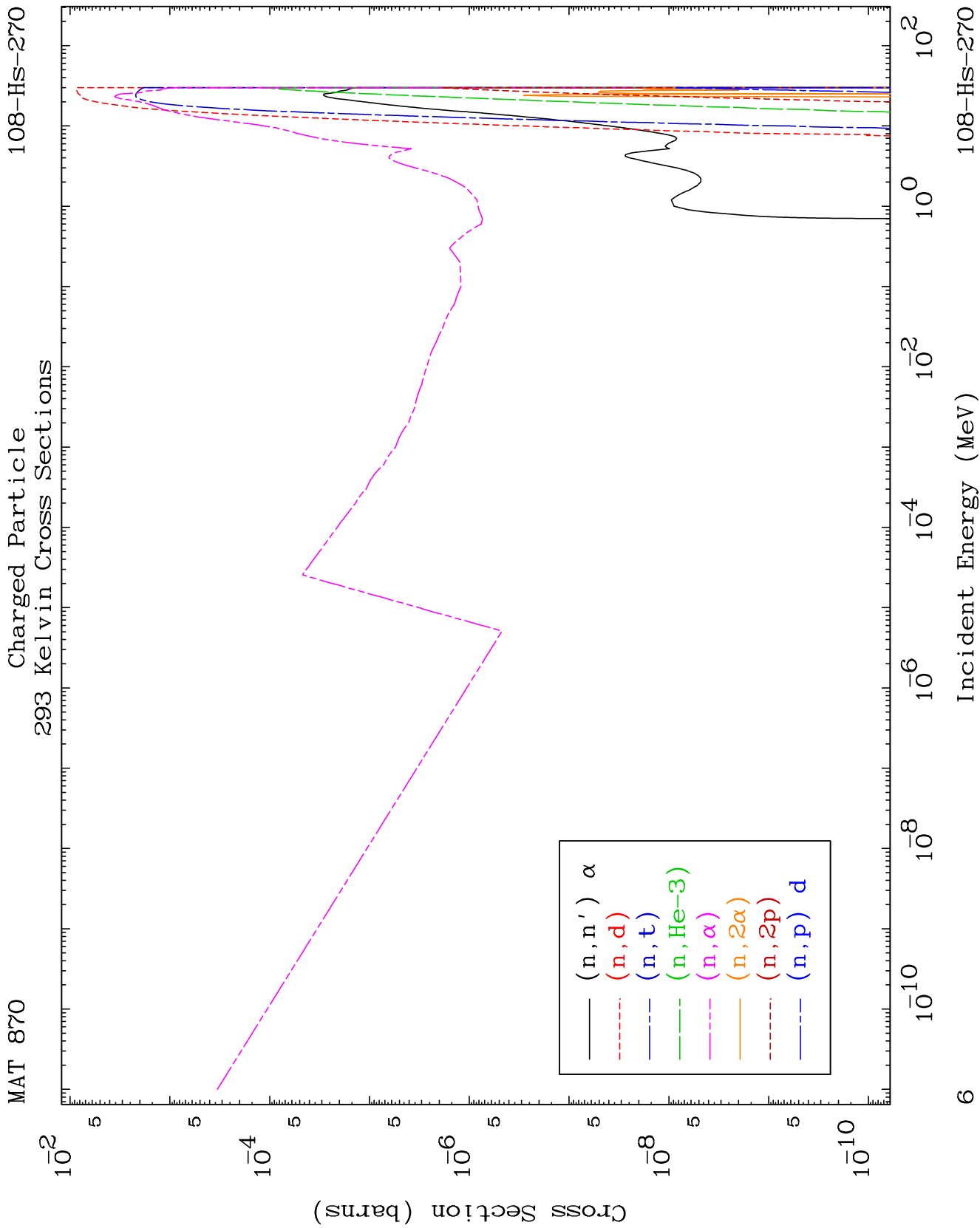
108-Hs-270



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Incident Energy (MeV)

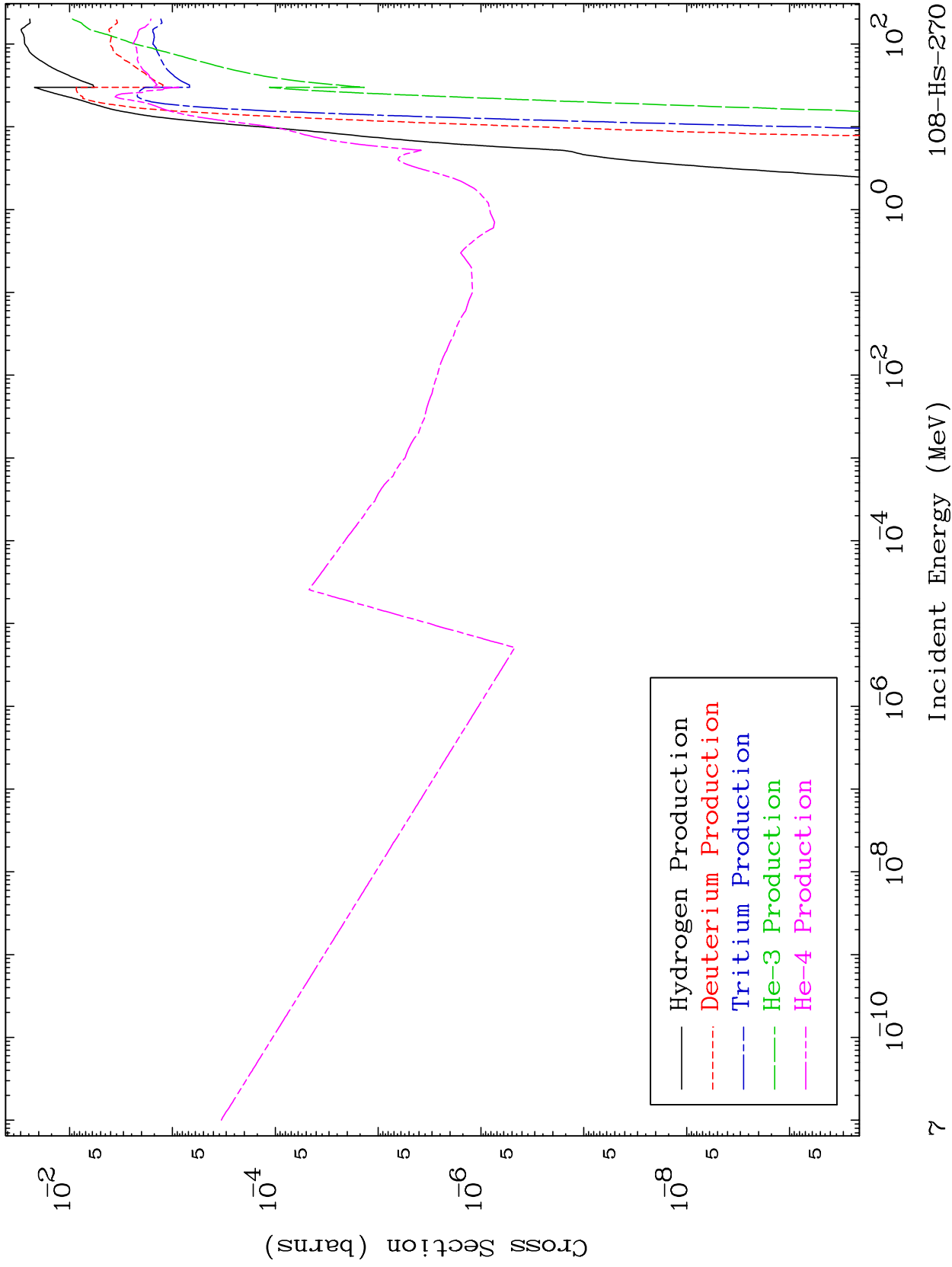
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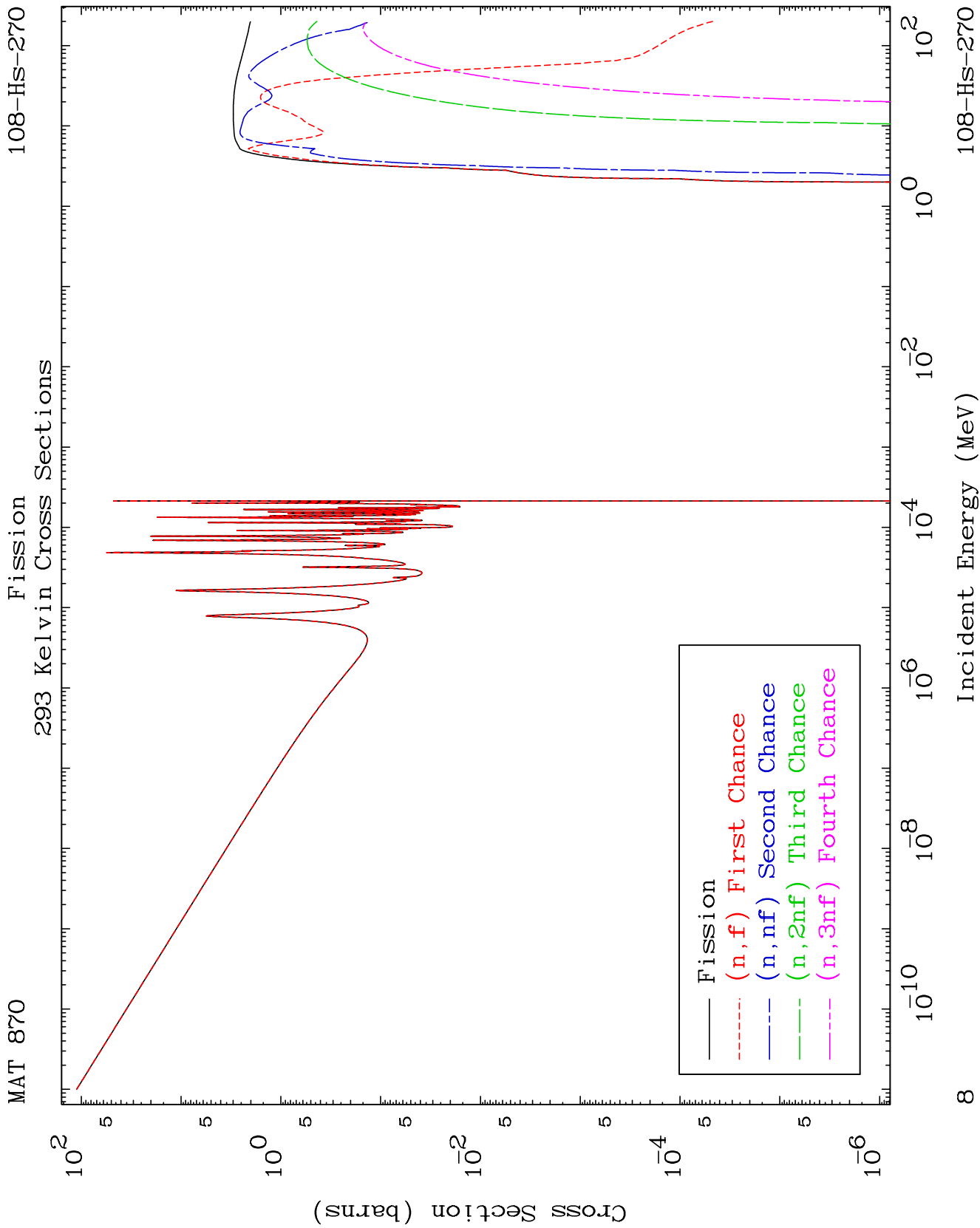


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Particle Production
293 Kelvin Cross Sections

108-Hs-270

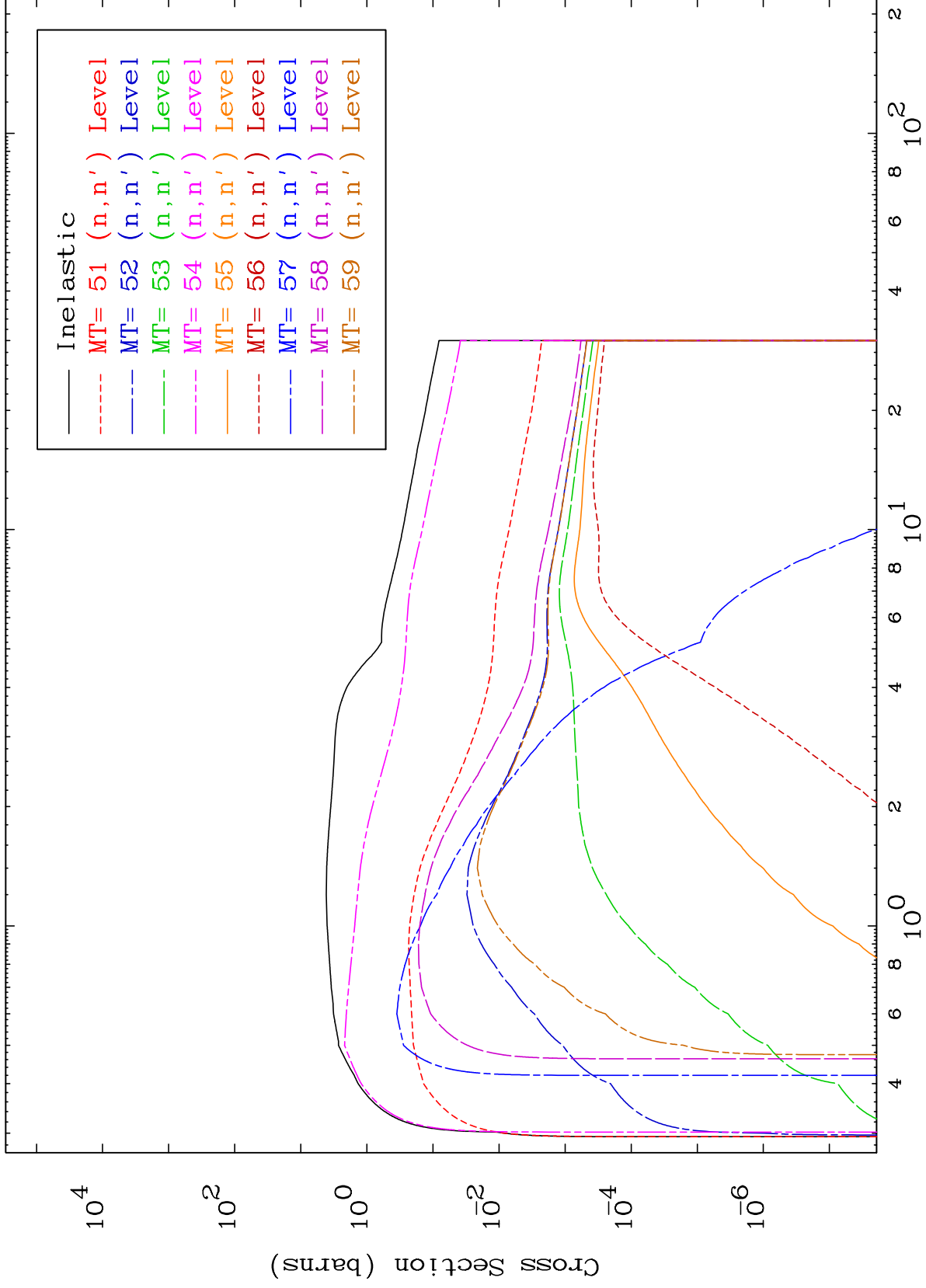




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(n,n') Levels
293 Kelvin Cross Sections

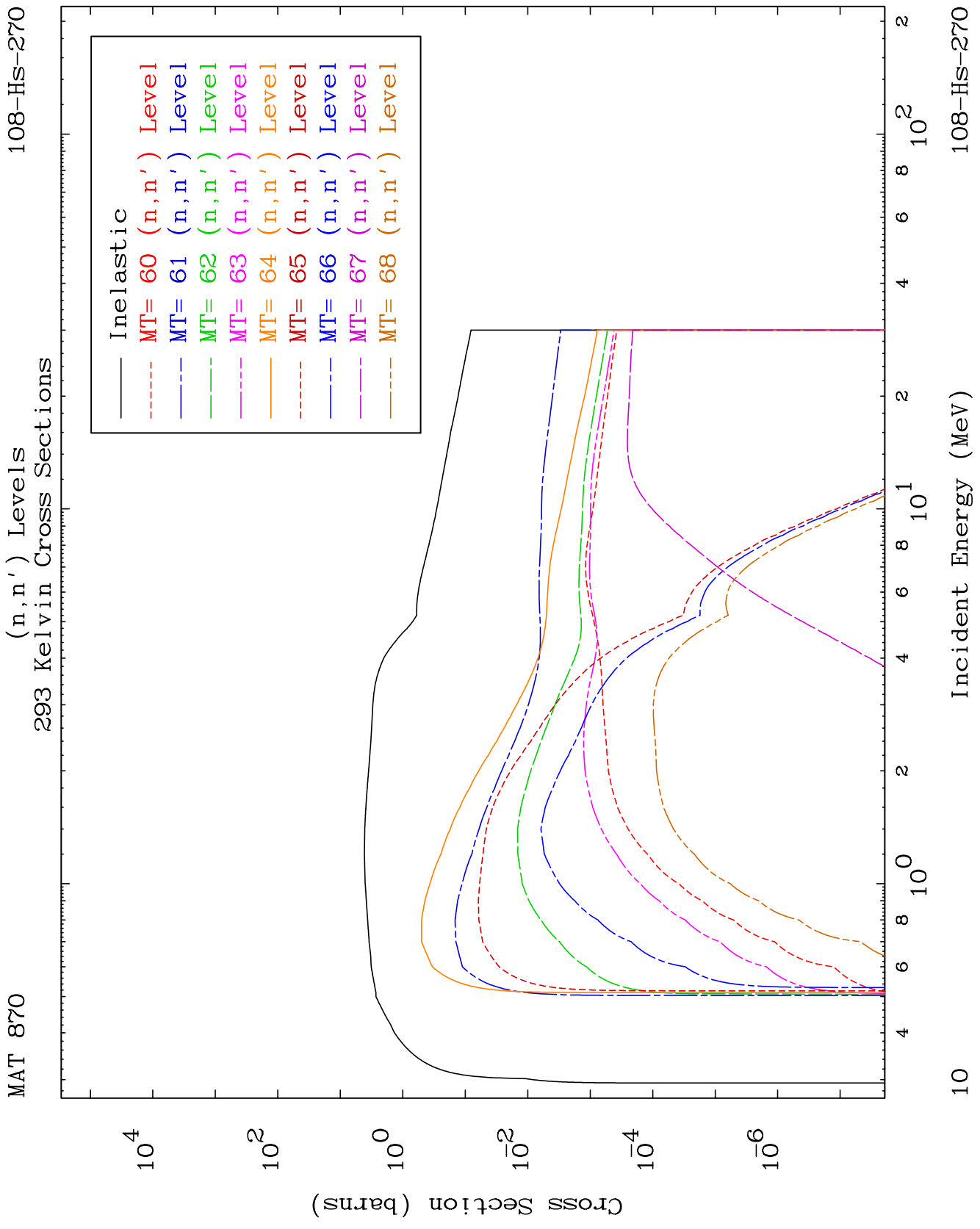
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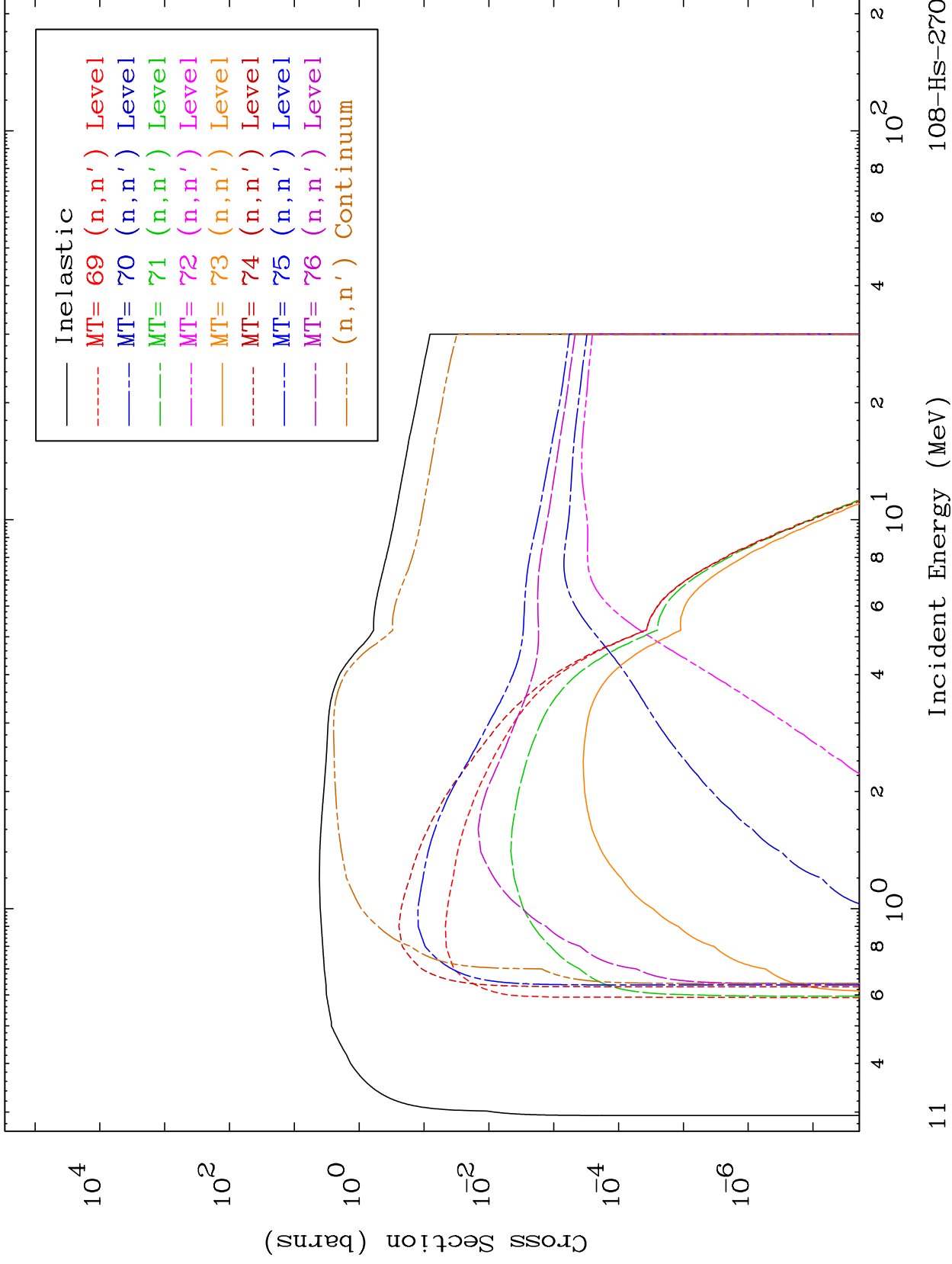


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Incident Energy (MeV)

108-Hs-270

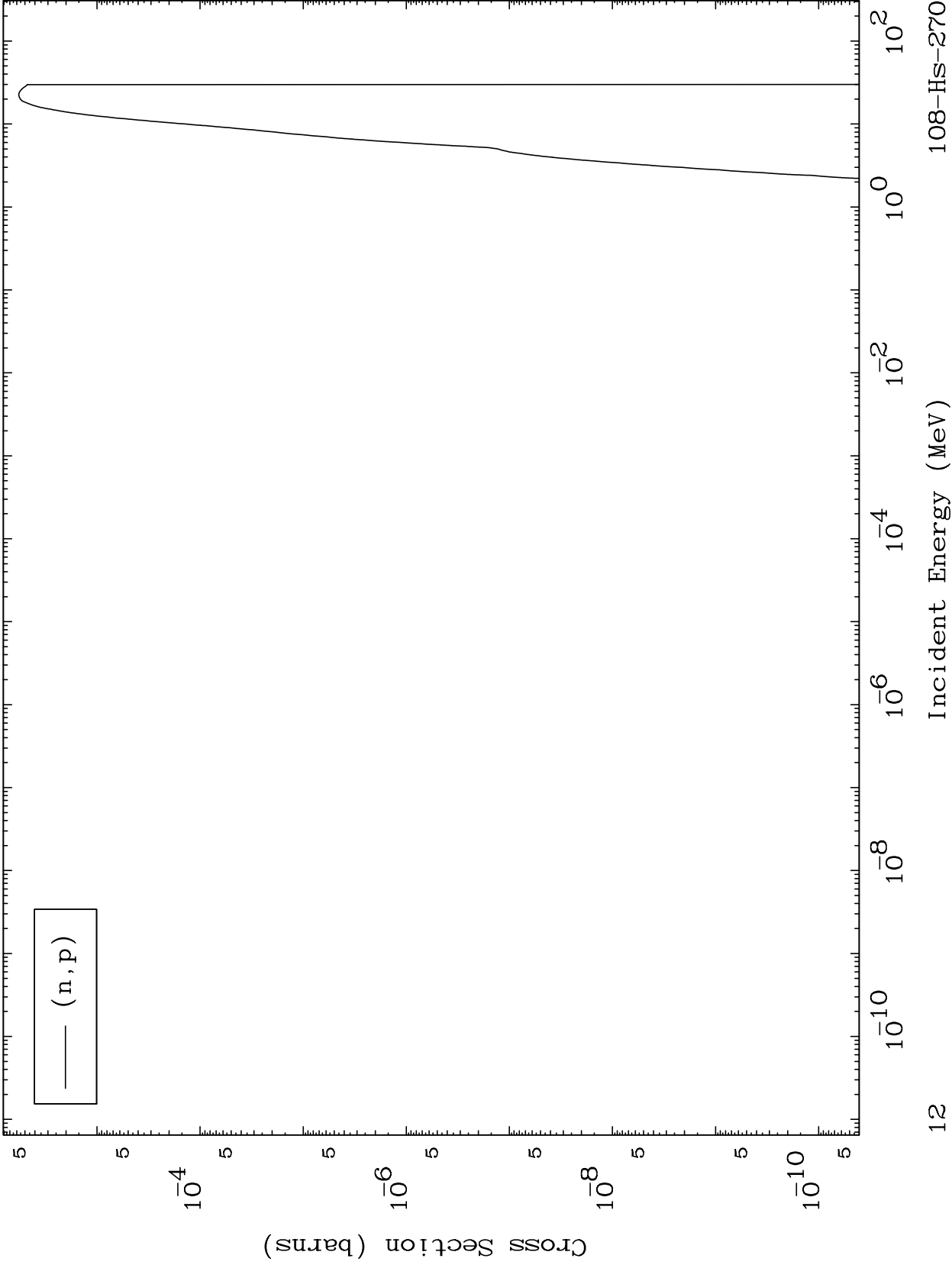


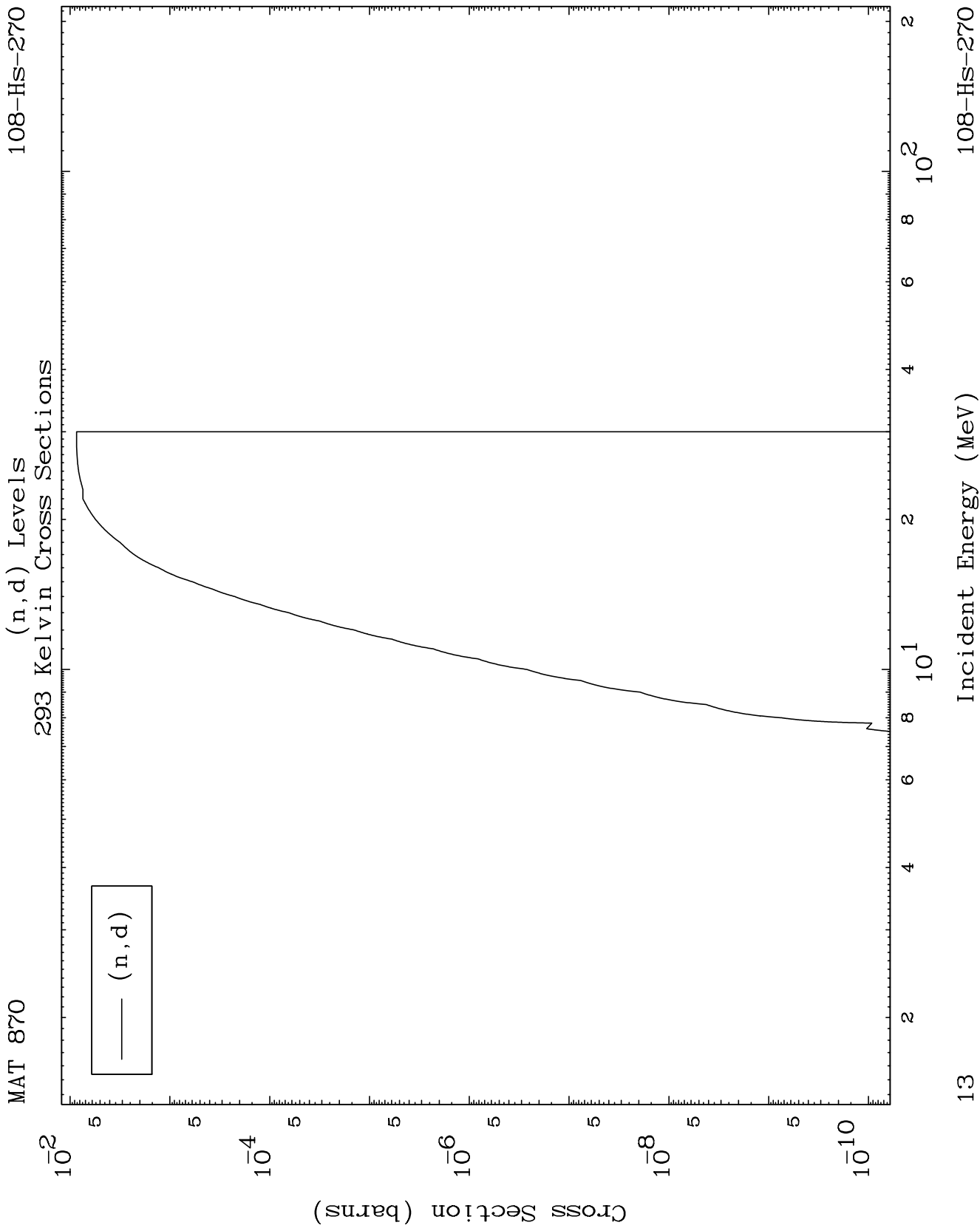


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(n,p) Levels
293 Kelvin Cross Sections

108-Hs-270

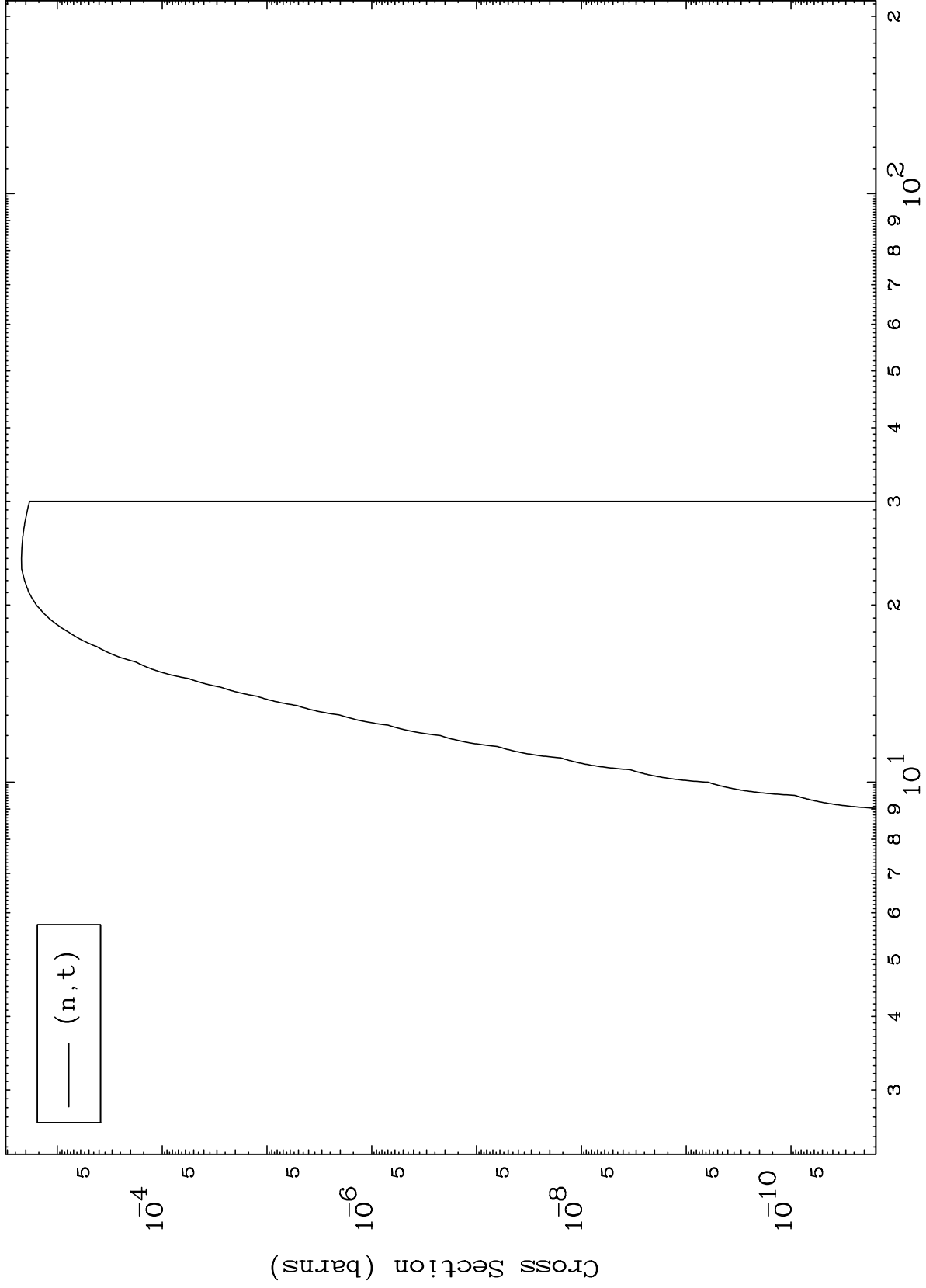




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(n,t) Levels
293 Kelvin Cross Sections

108-Hs-270



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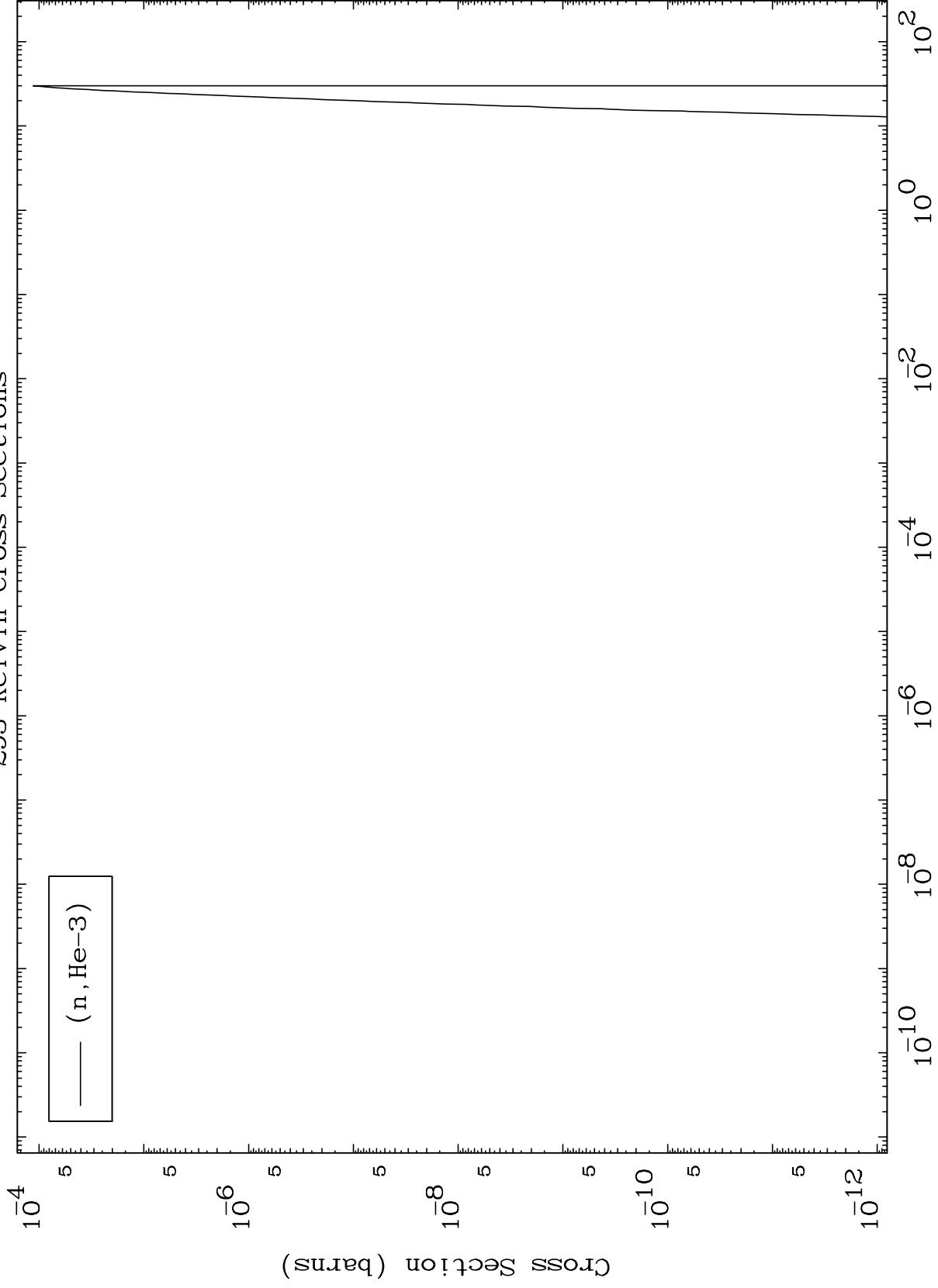
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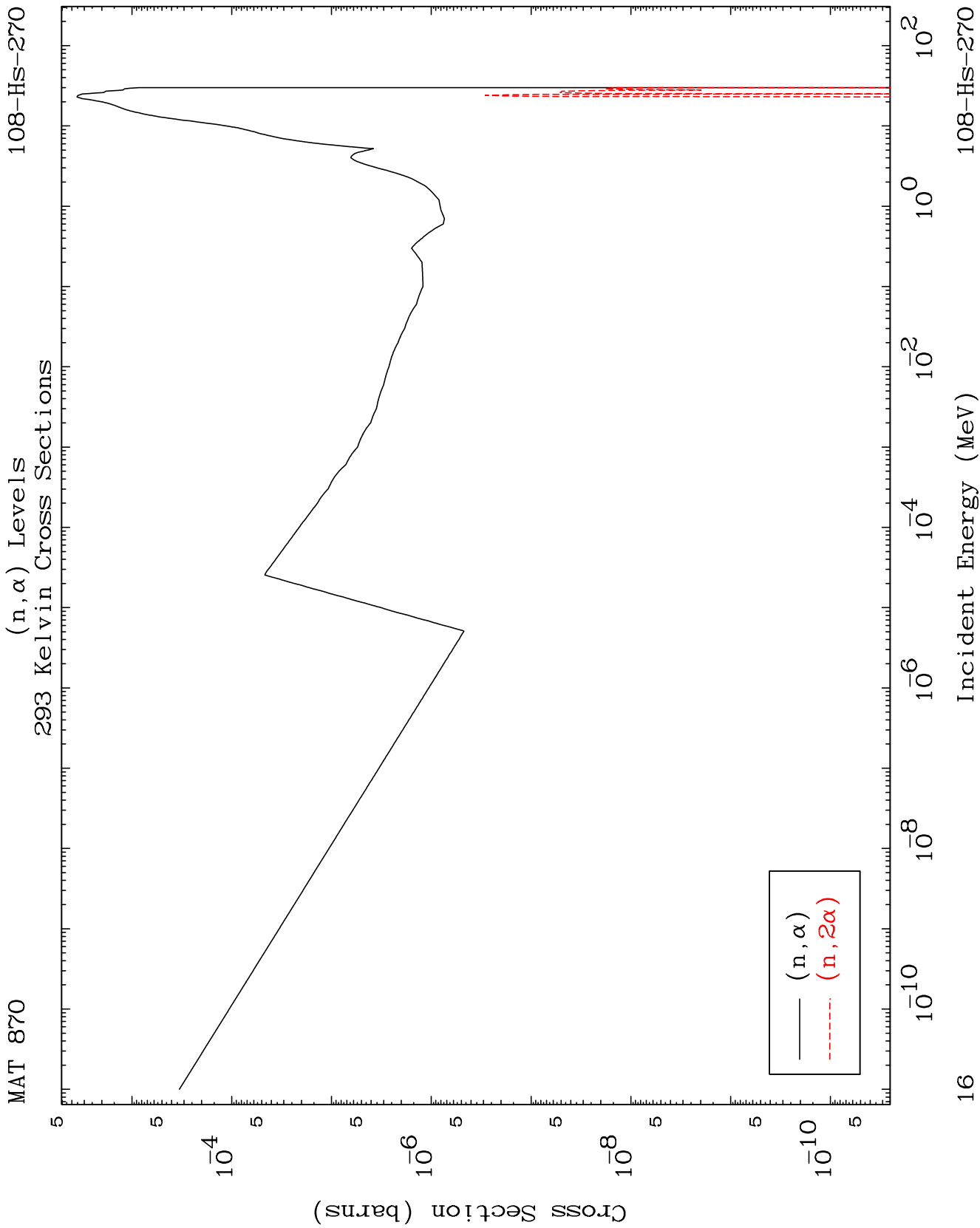
108-Hs-270

MAT 870

(n,He3) Levels
293 Kelvin Cross Sections

108-Hs-270

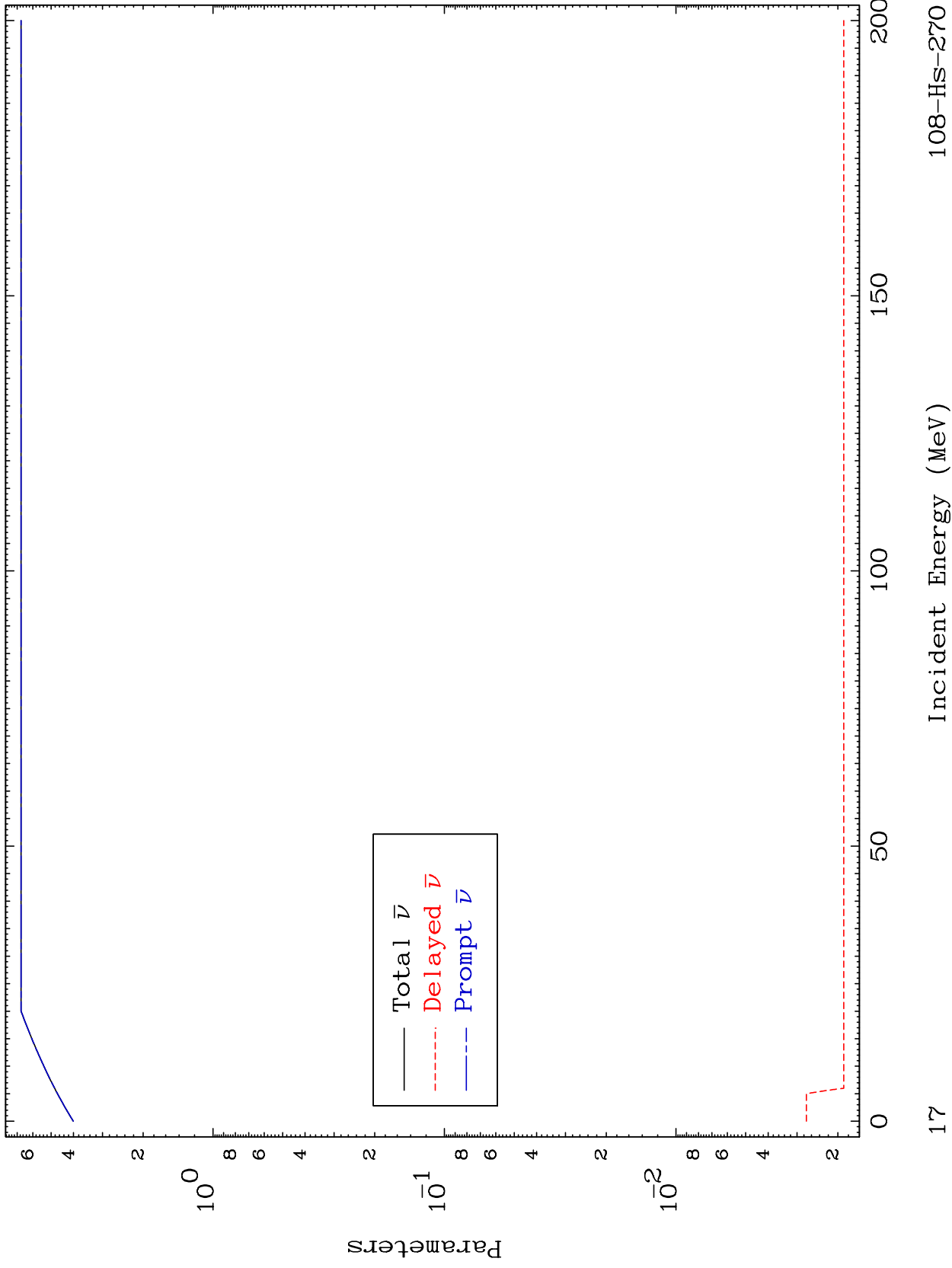




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Energy Release
Parameters

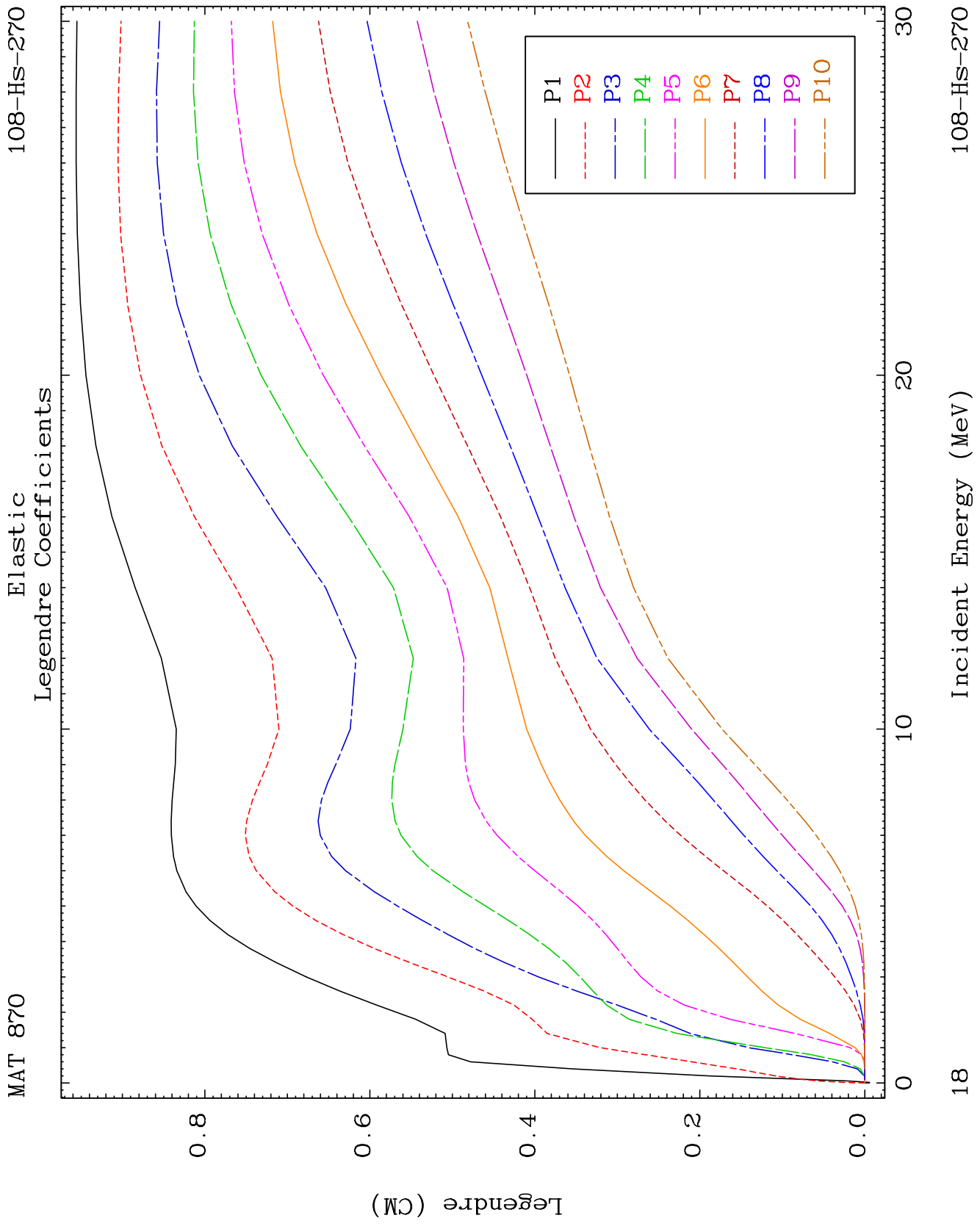
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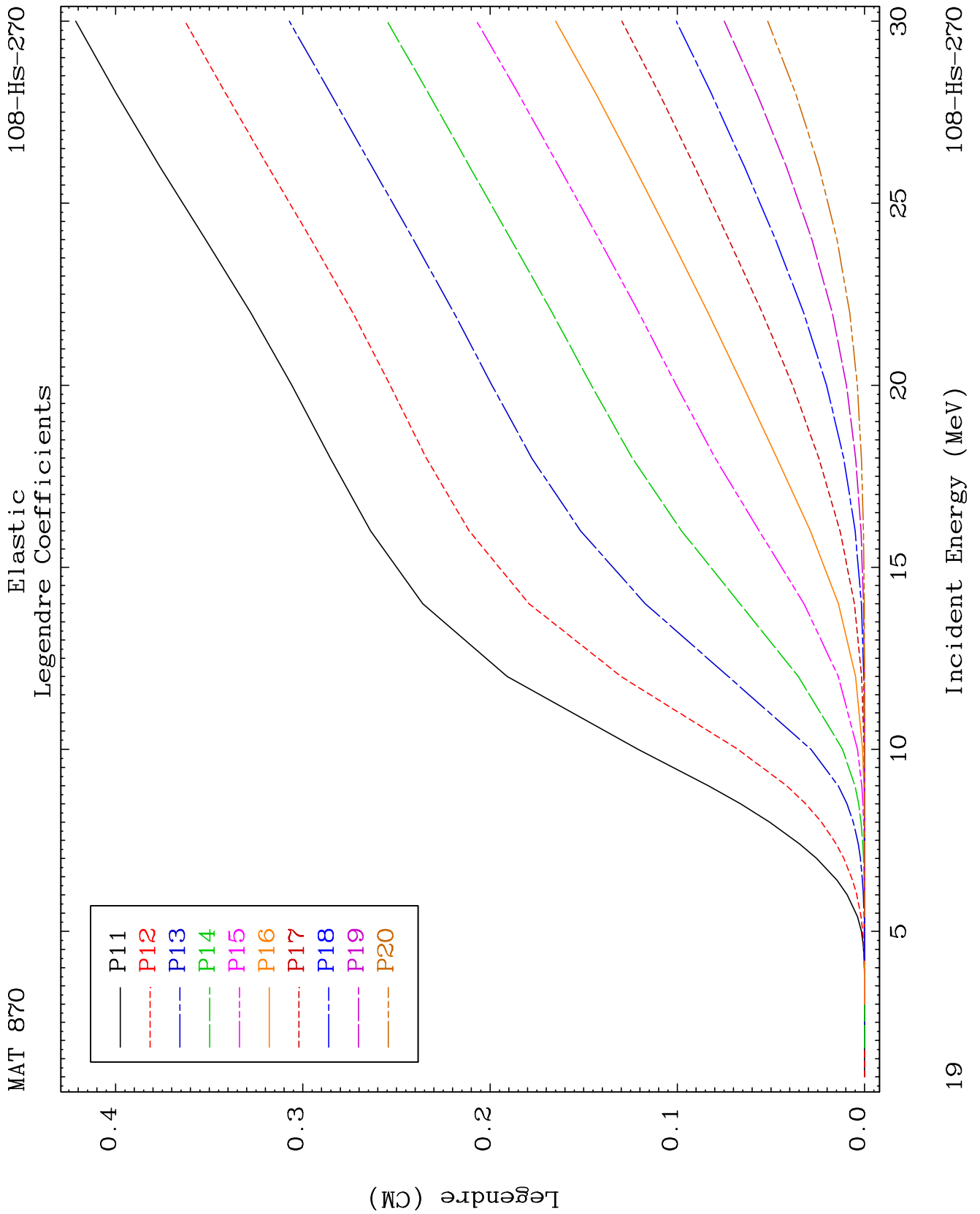


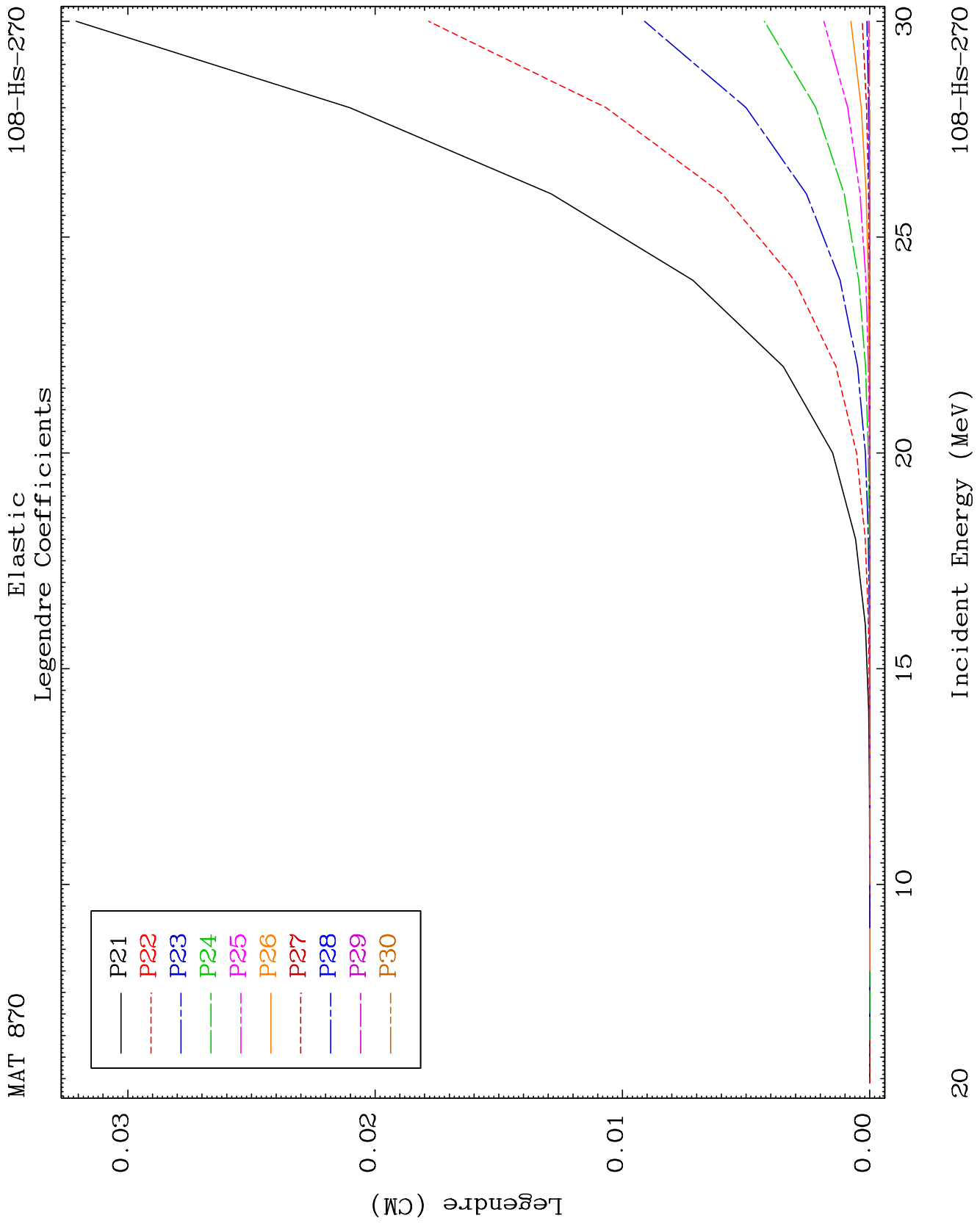
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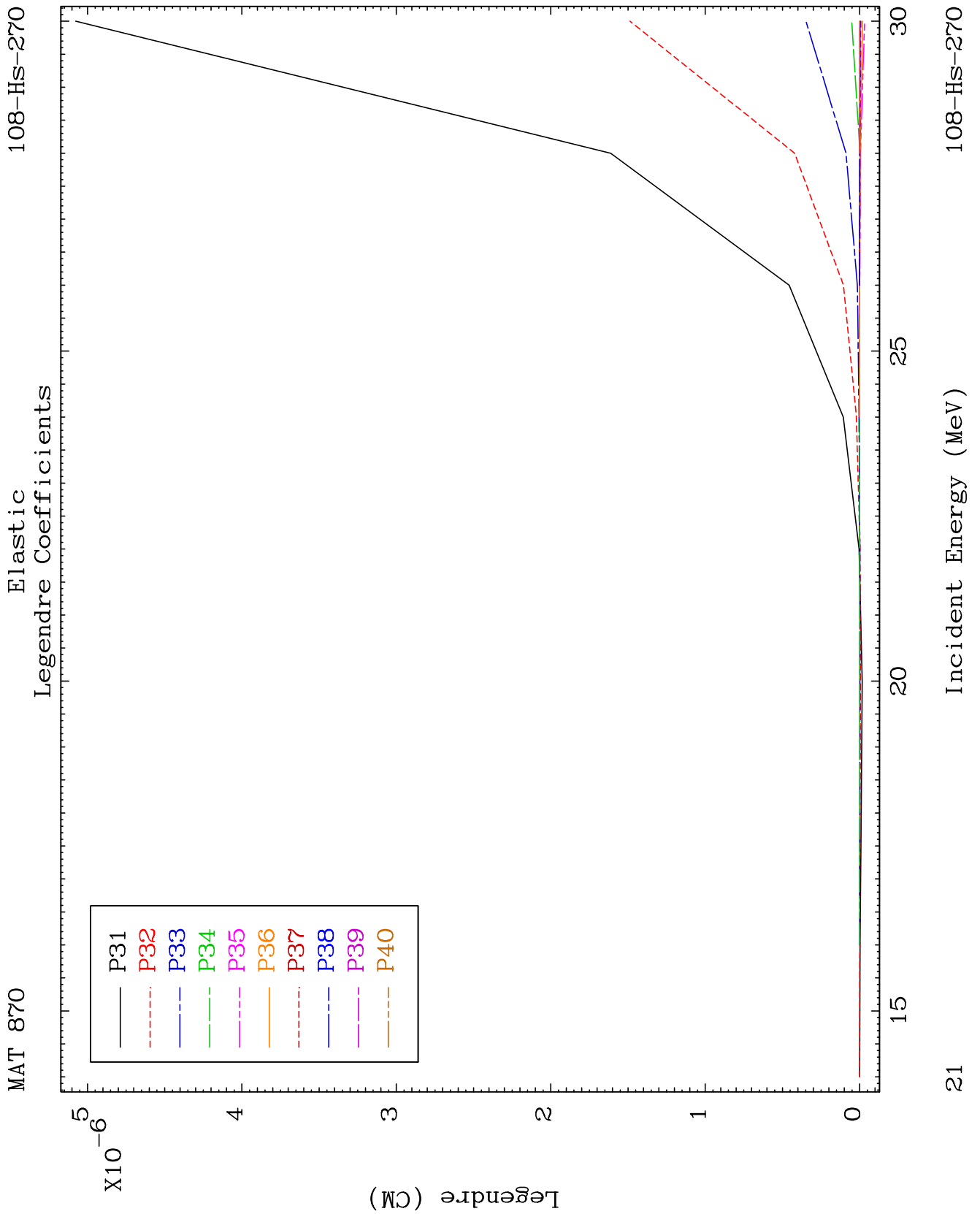
Incident Energy (MeV)

108-Hs-270





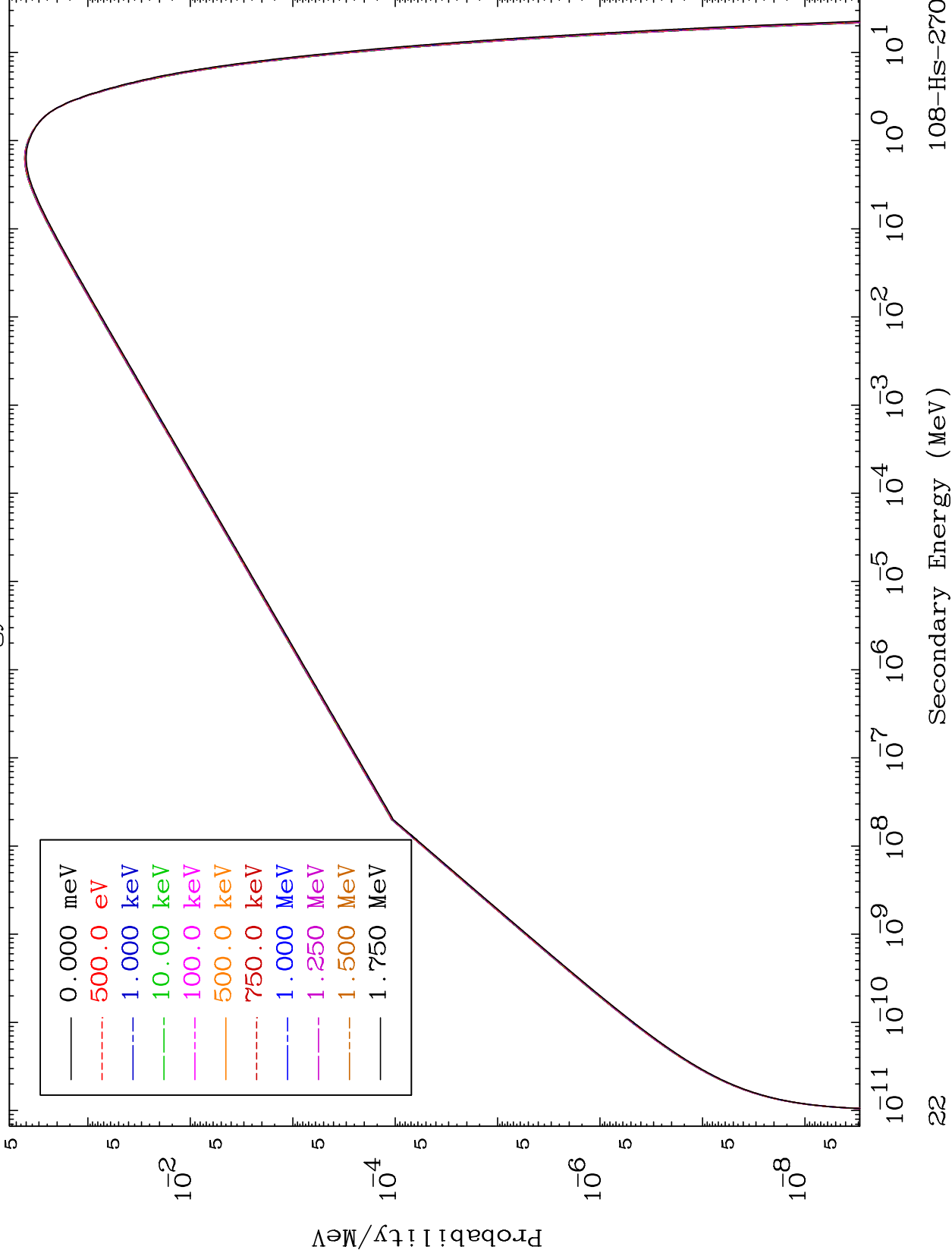




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Fission Energy Distributions

108-Hs-270



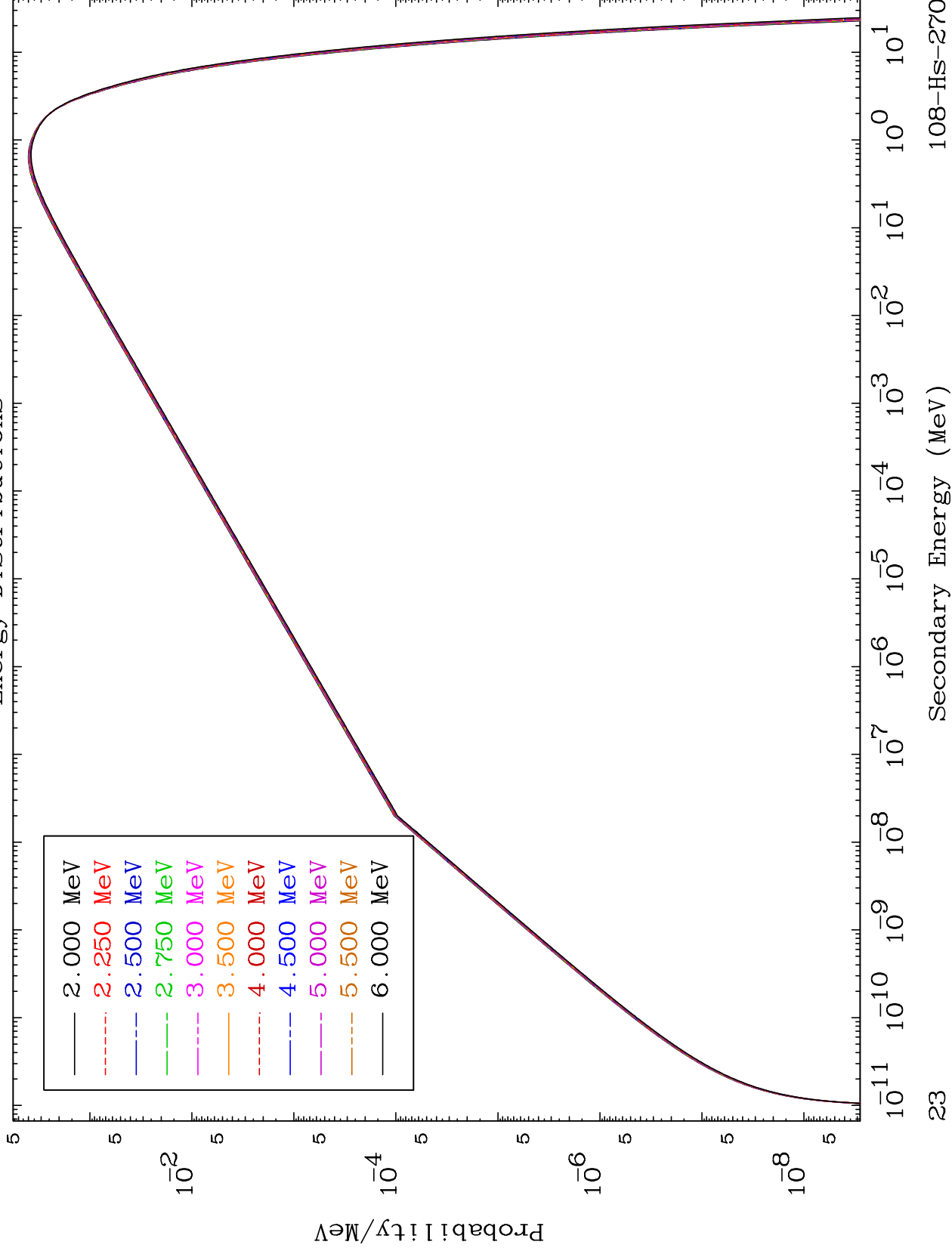
108-Hs-270

Secondary Energy (MeV)

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Fission Energy Distributions

108-Hs-270



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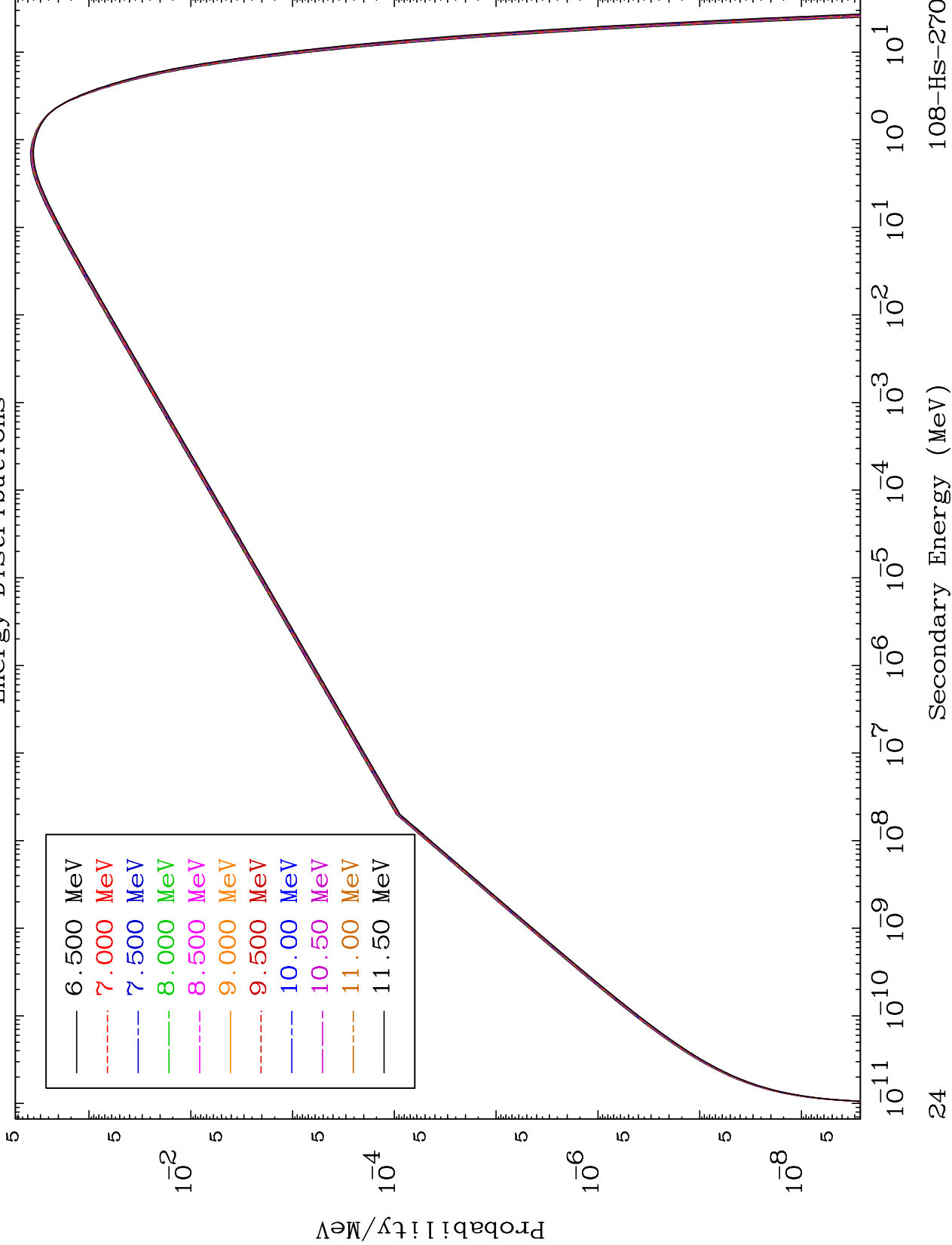
Secondary Energy (MeV)

108-Hs-270

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Fission Energy Distributions

108-Hs-270



108-Hs-270

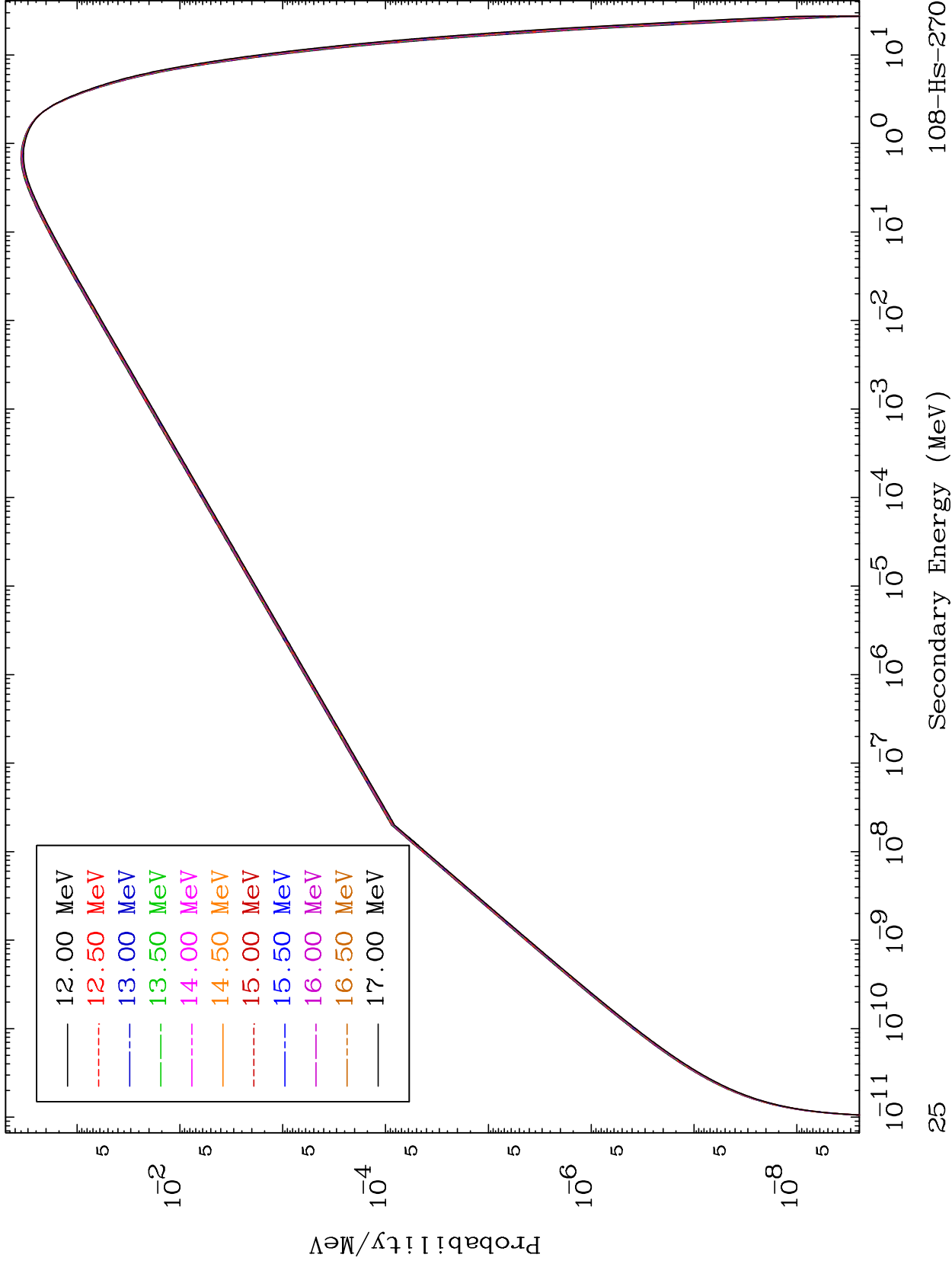
Secondary Energy (MeV)

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Fission Energy Distributions

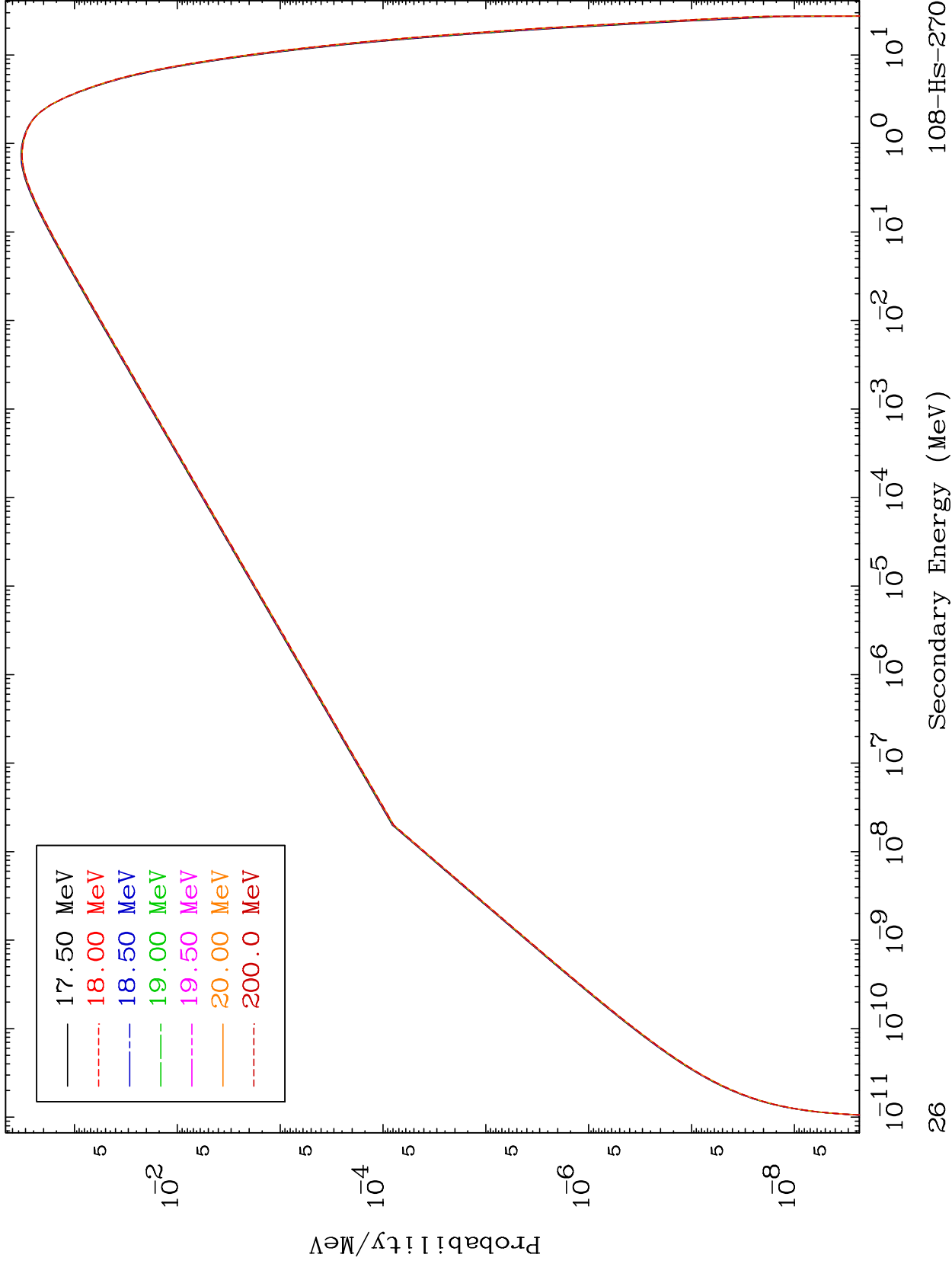
108-Hs-270



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Fission Energy Distributions

108-Hs-270



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