

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail: [redcullen1@comcast.net](mailto:redcullen1@comcast.net)

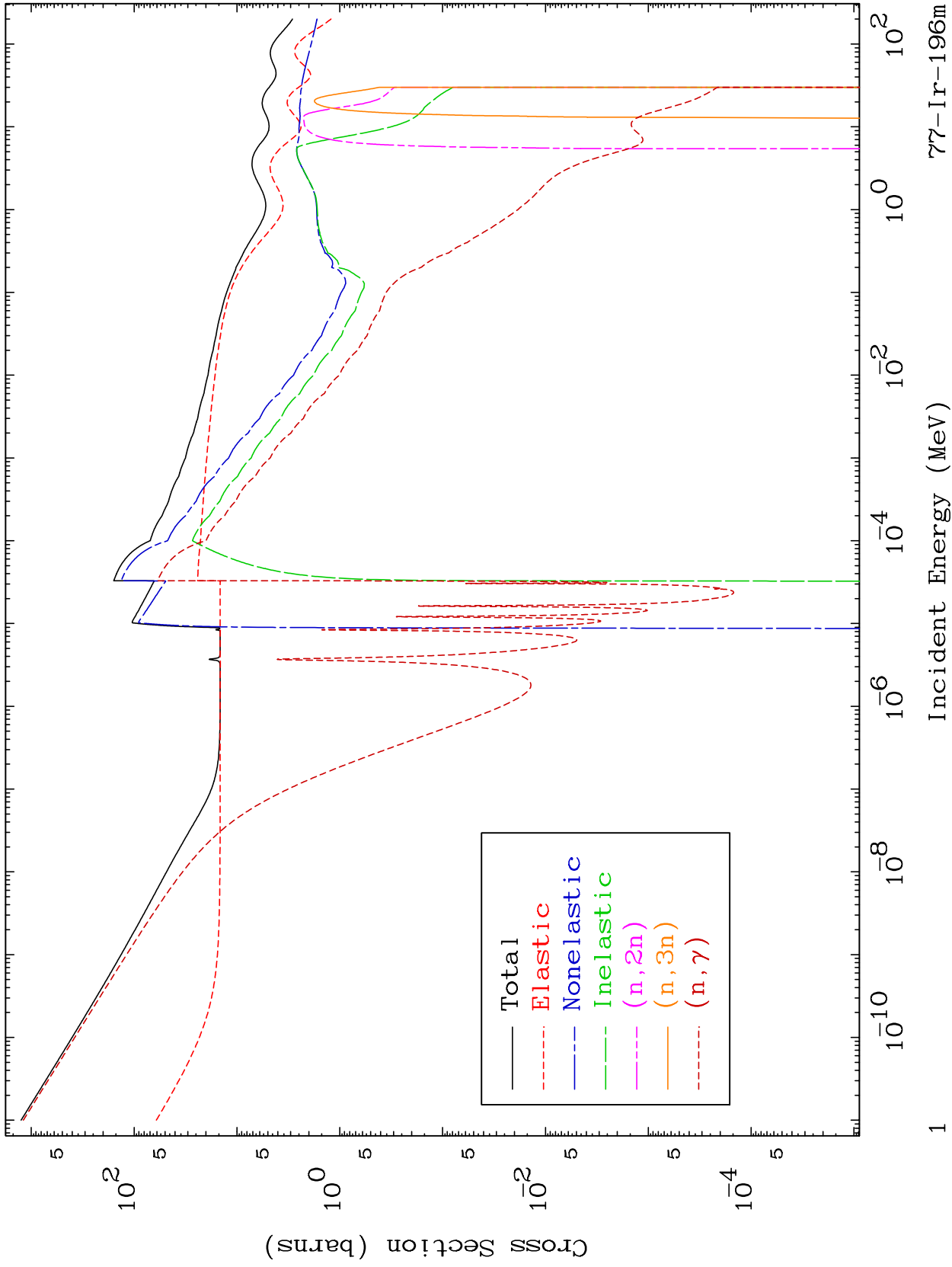
Web: [redcullen1.net/HOMEPAGE.NEW](http://redcullen1.net/HOMEPAGE.NEW)

Press Mouse Button to Start

MAT 7741

Neutron Major  
293 Kelvin Cross Sections

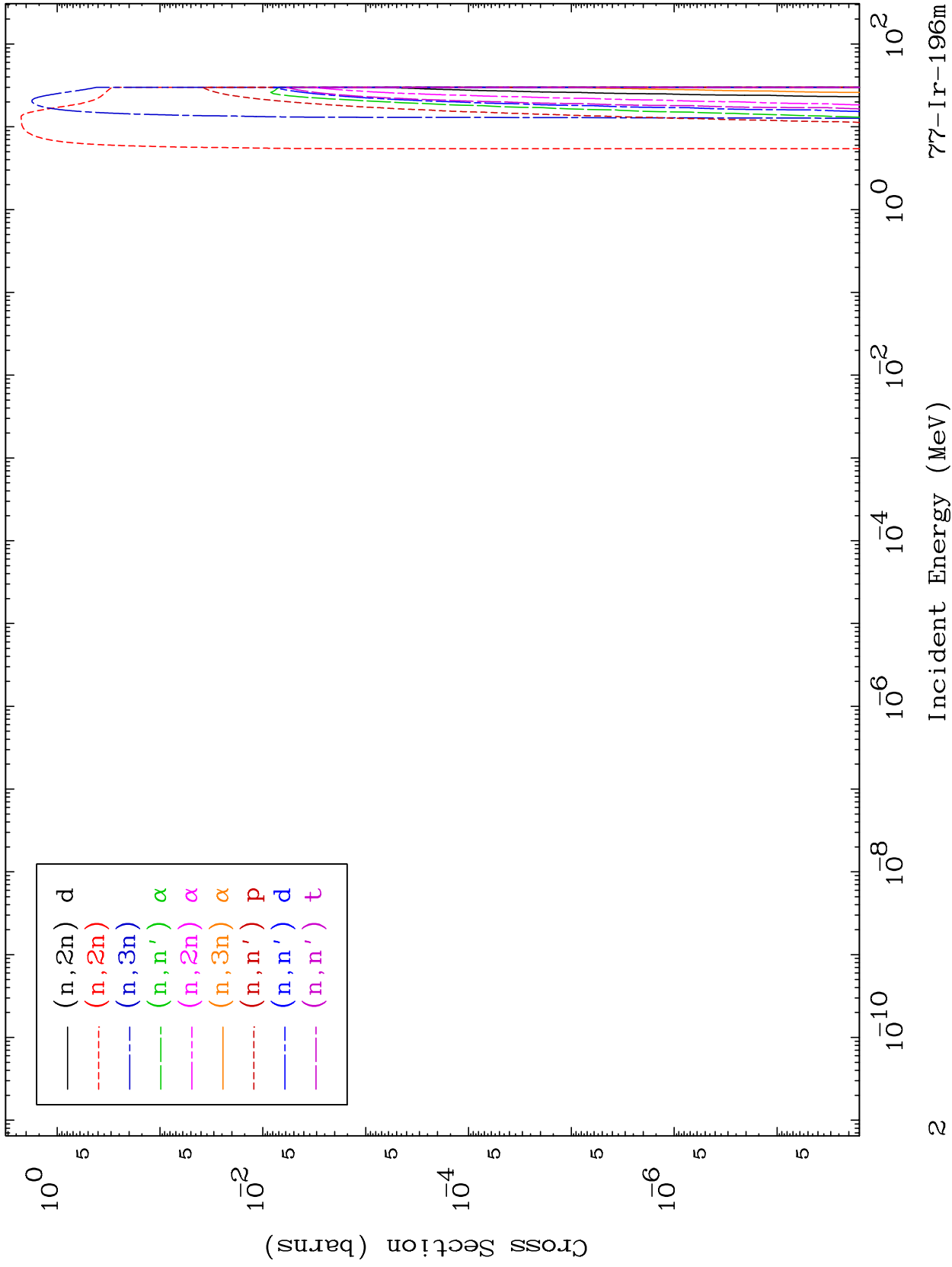
77-Ir-196m



MAT 7741

Neutron Absorption  
293 Kelvin Cross Sections

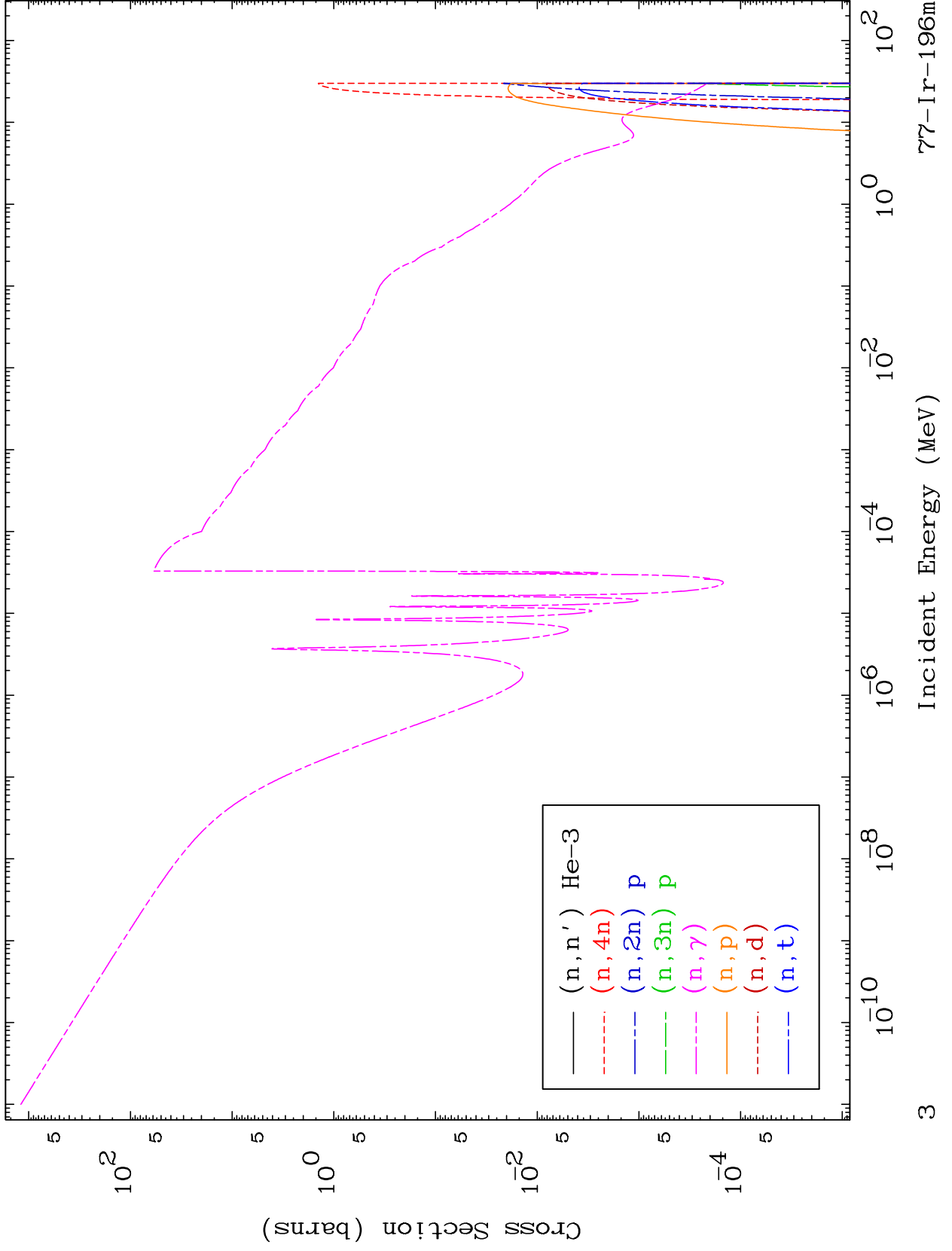
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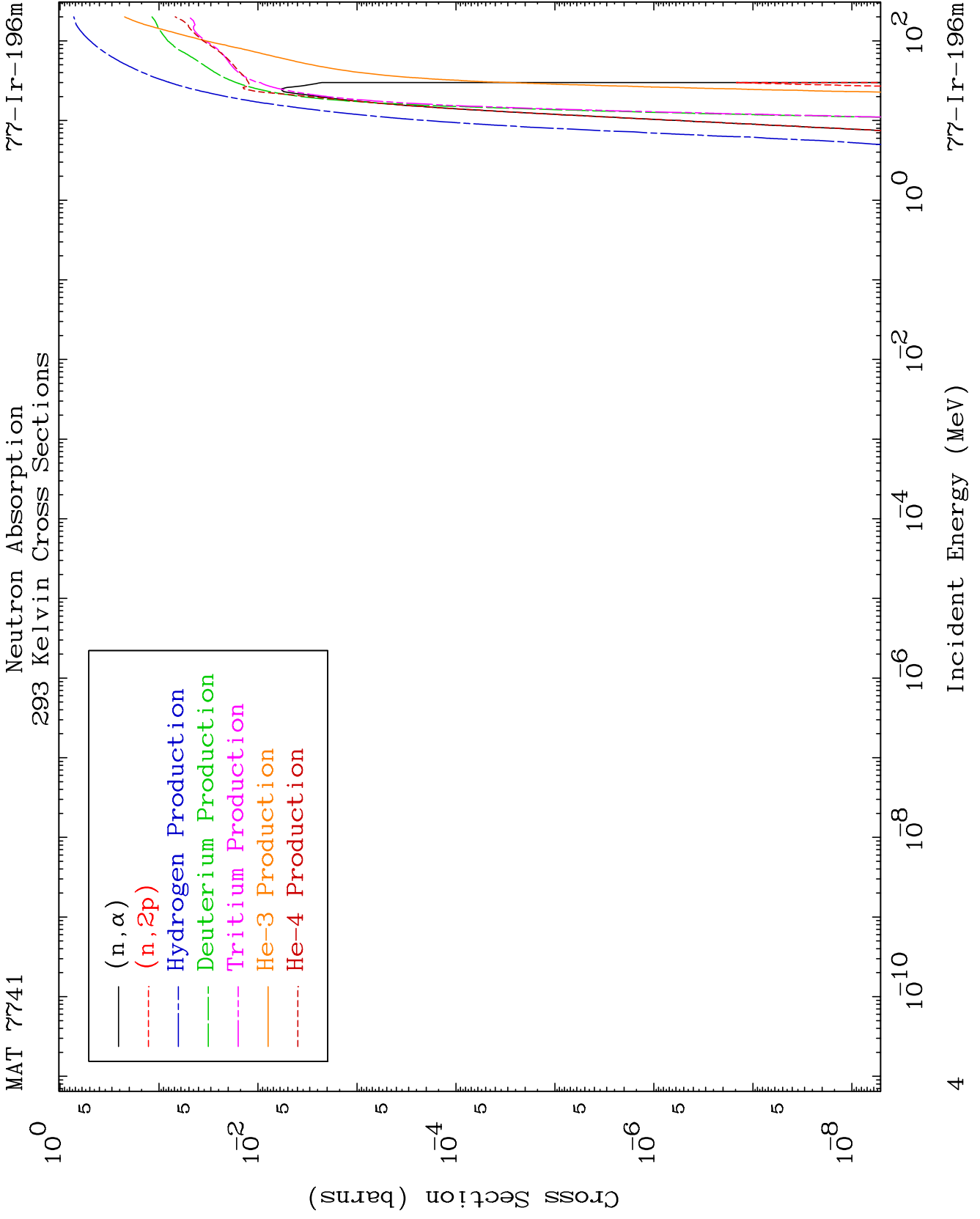


MAT 7741

Neutron Absorption  
293 Kelvin Cross Sections

<sup>77</sup>Ir-196m

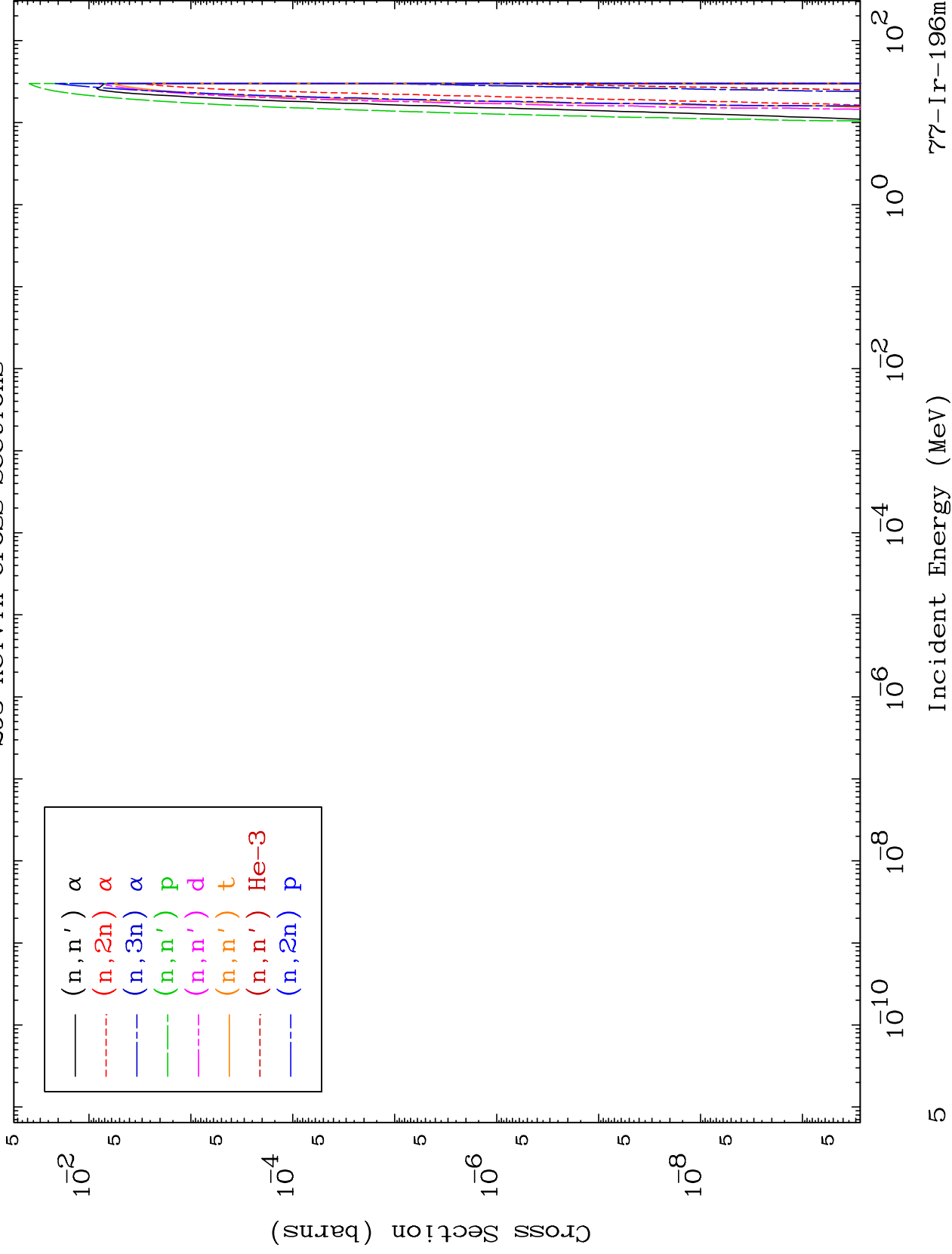




MAT 7741

Charged Particle  
293 Kelvin Cross Sections

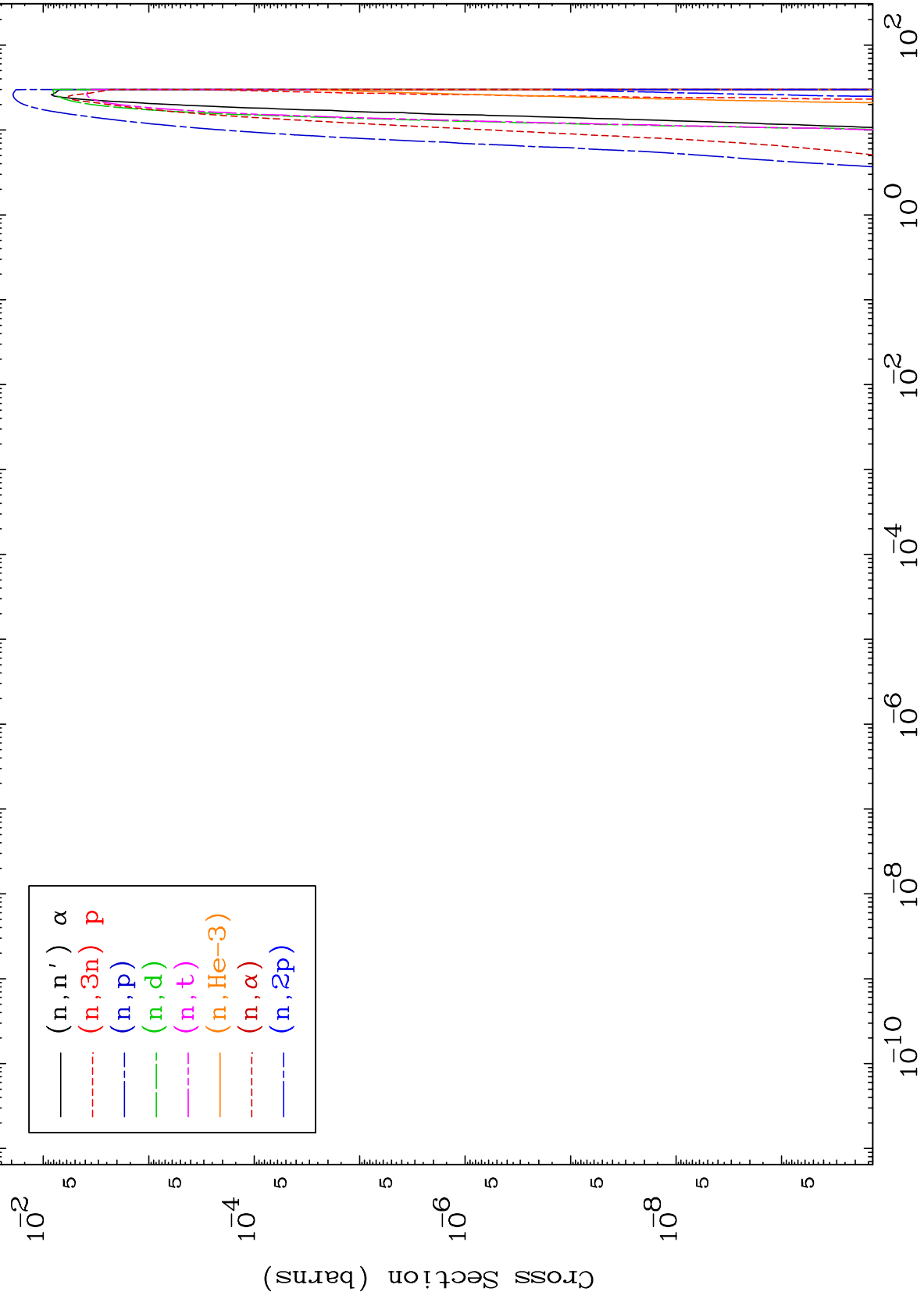
77-Ir-196m



MAT 7741

Charged Particle  
293 Kelvin Cross Sections

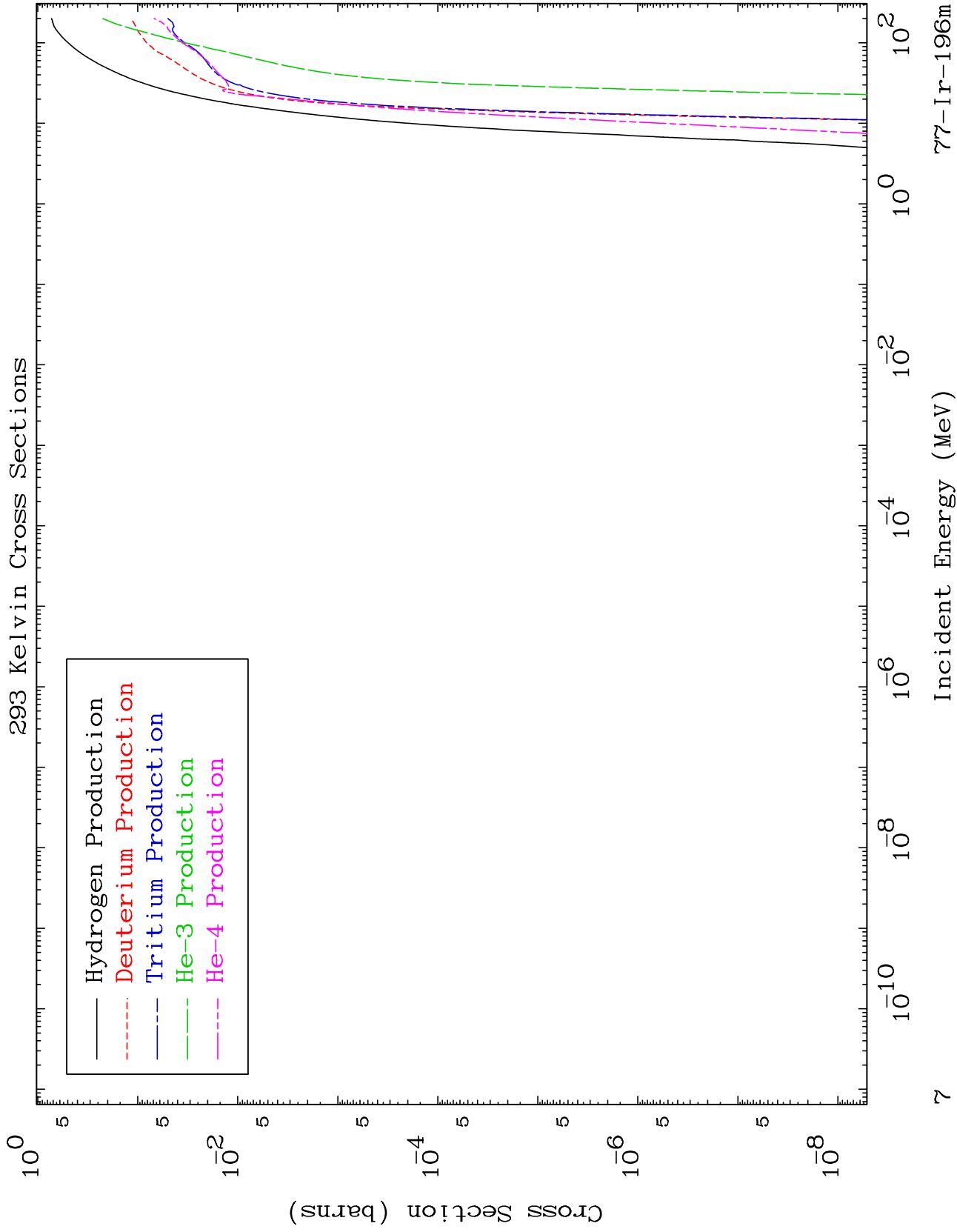
77-Ir-196m



MAT 7741

Particle Production  
293 Kelvin Cross Sections

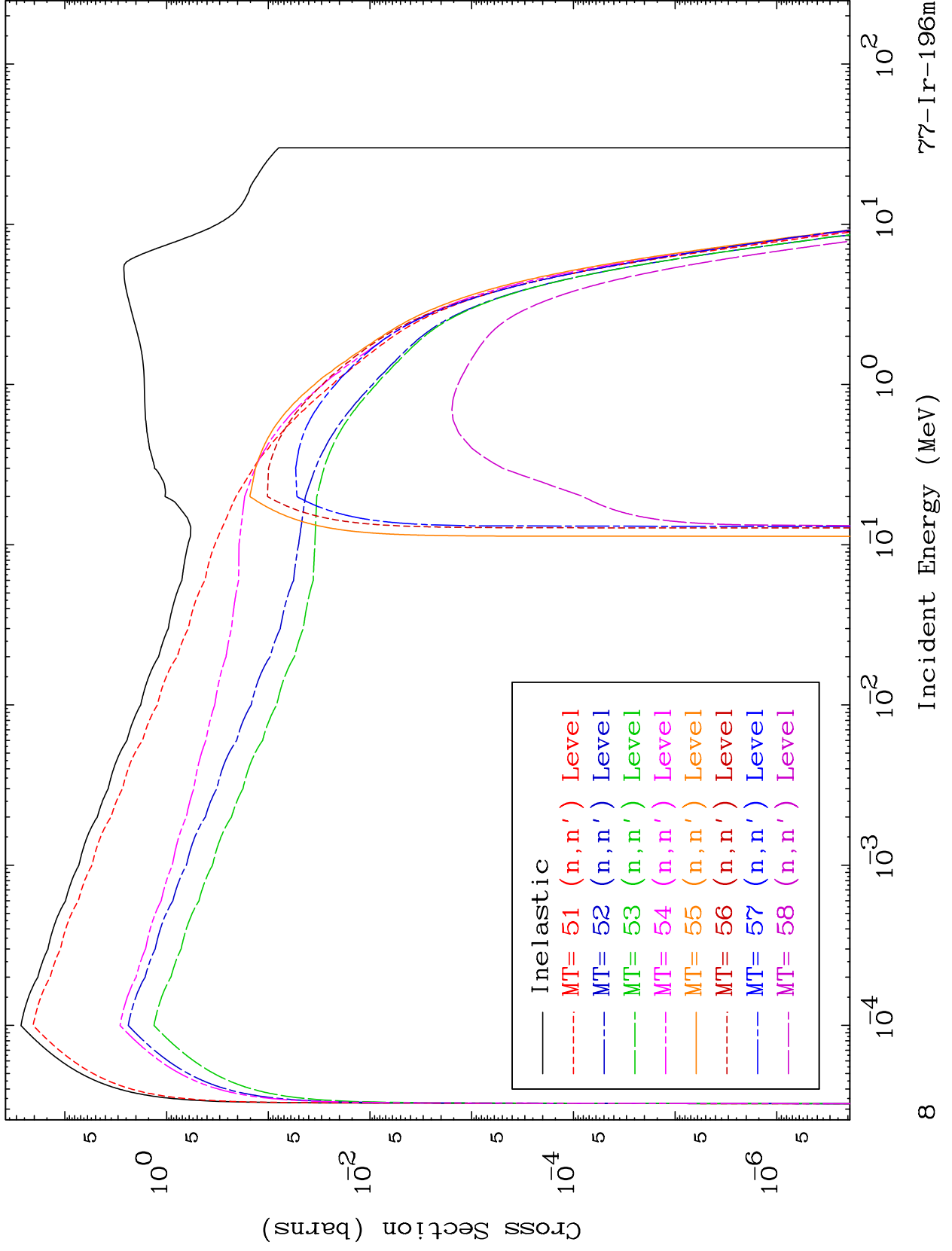
77-Ir-196m



MAT 7741

(n,n') Levels  
293 Kelvin Cross Sections

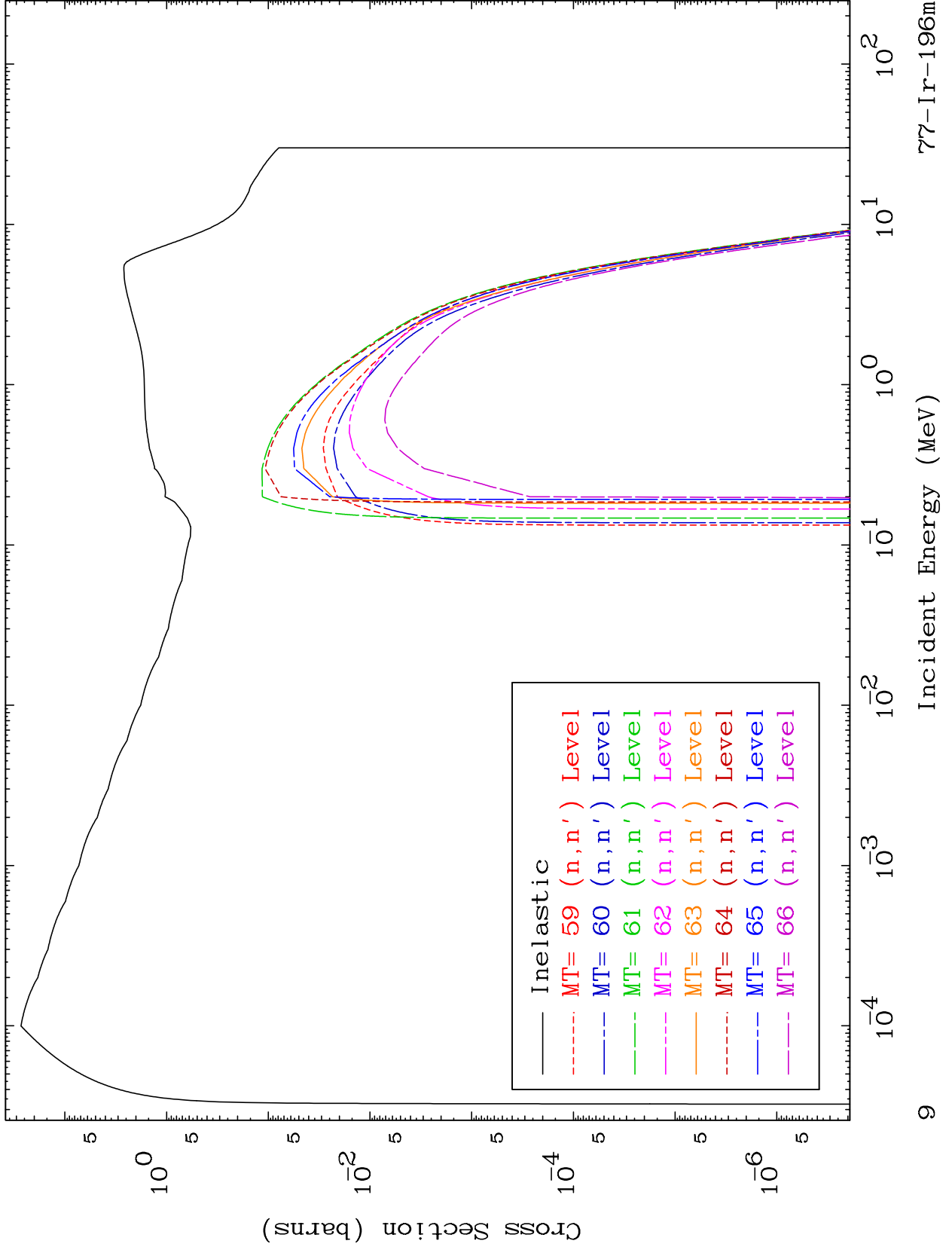
77-Ir-196m



MAT 7741

(n,n') Levels  
293 Kelvin Cross Sections

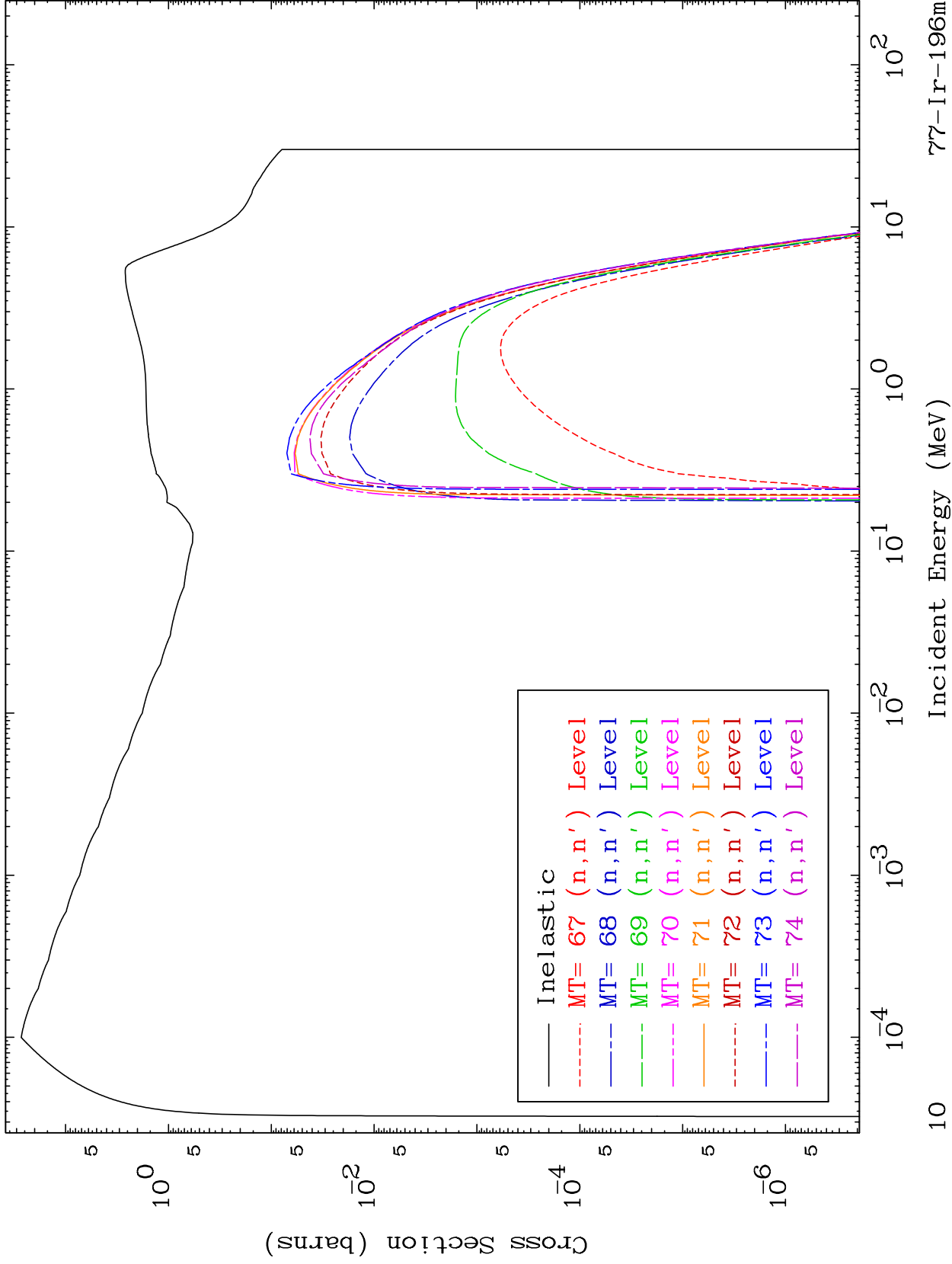
77-Ir-196m



MAT 7741

(n,n') Levels  
293 Kelvin Cross Sections

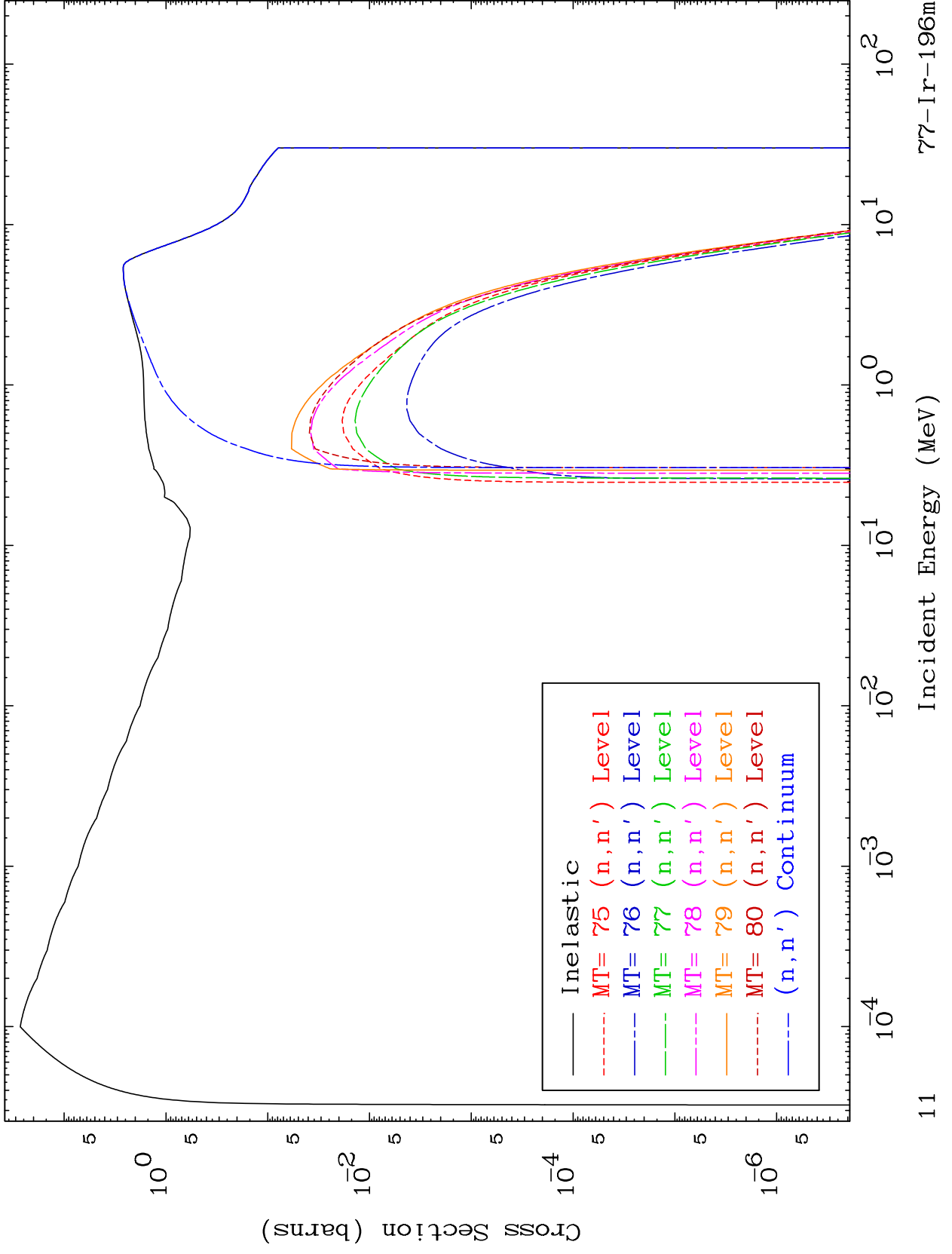
77-Ir-196m



MAT 7741

(n,n') Levels  
293 Kelvin Cross Sections

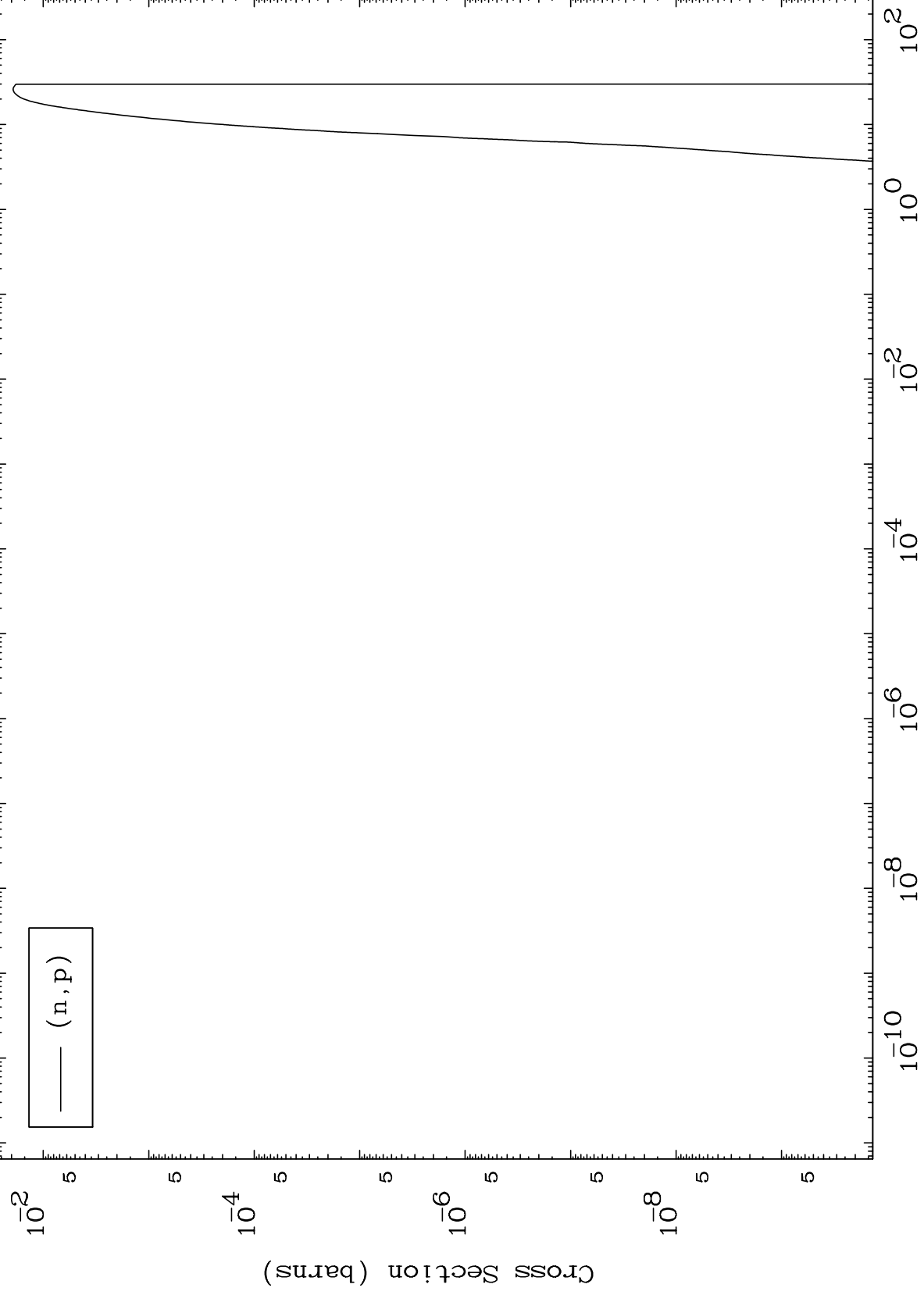
77-Ir-196m



MAT 7741

(n,p) Levels  
293 Kelvin Cross Sections

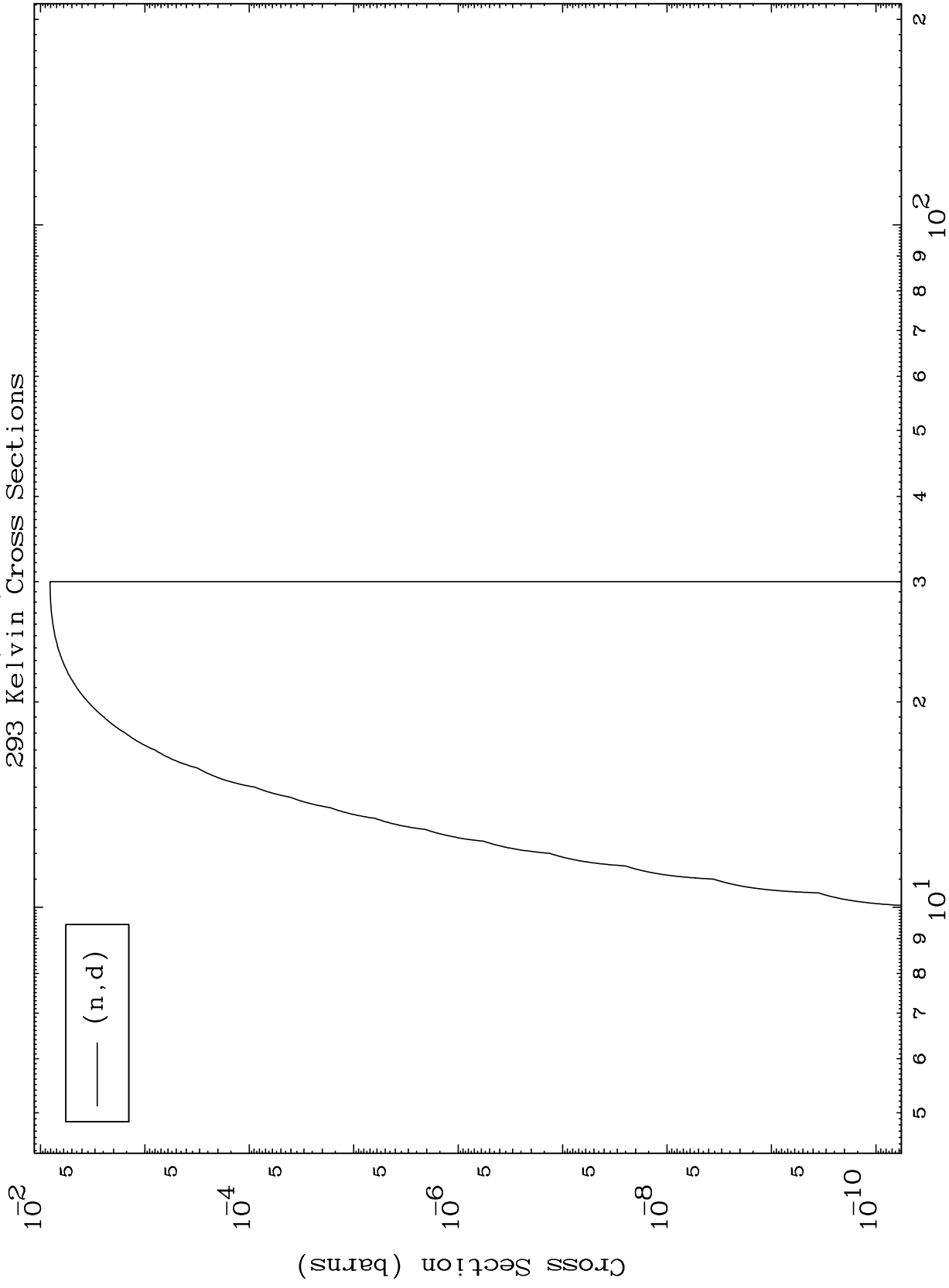
77-Ir-196m



MAT 7741

(n,d) Levels  
293 Kelvin Cross Sections

77-Ir-196m



13

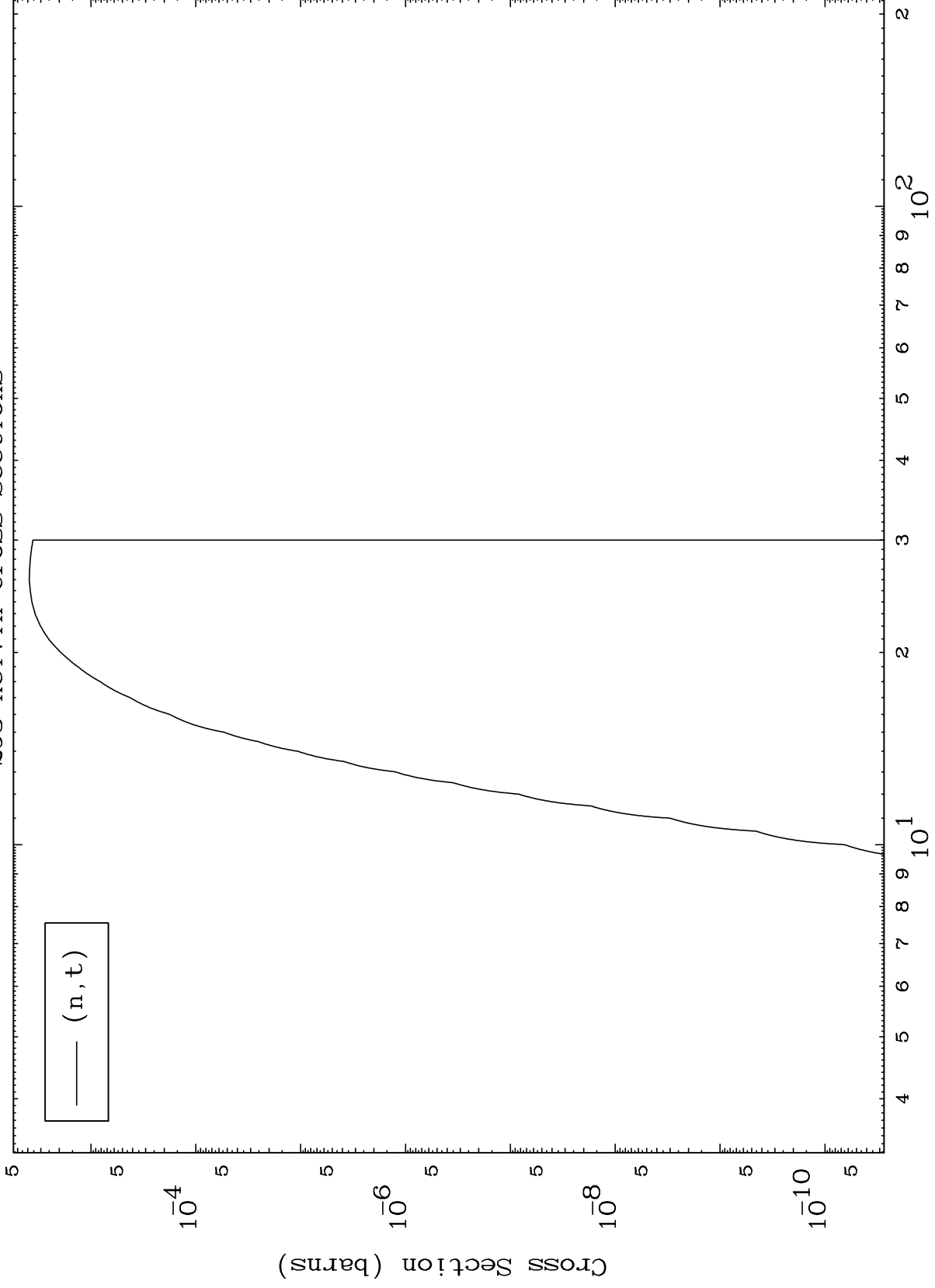
Incident Energy (MeV)

77-Ir-196m

MAT 7741

(n,t) Levels  
293 Kelvin Cross Sections

77-Ir-196m



14

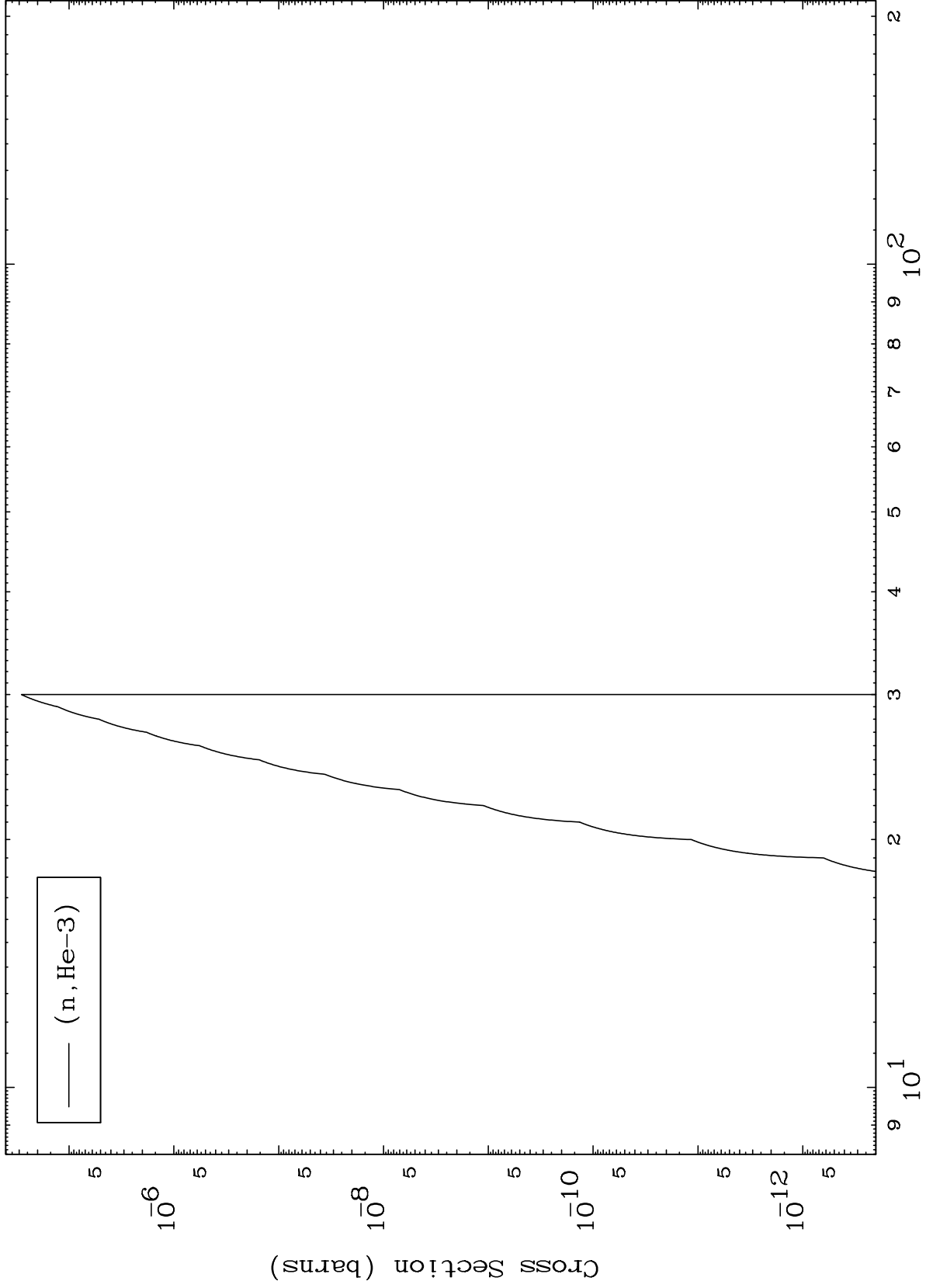
Incident Energy (MeV)

77-Ir-196m

MAT 7741

(n,He3) Levels  
293 Kelvin Cross Sections

77-Ir-196m



15

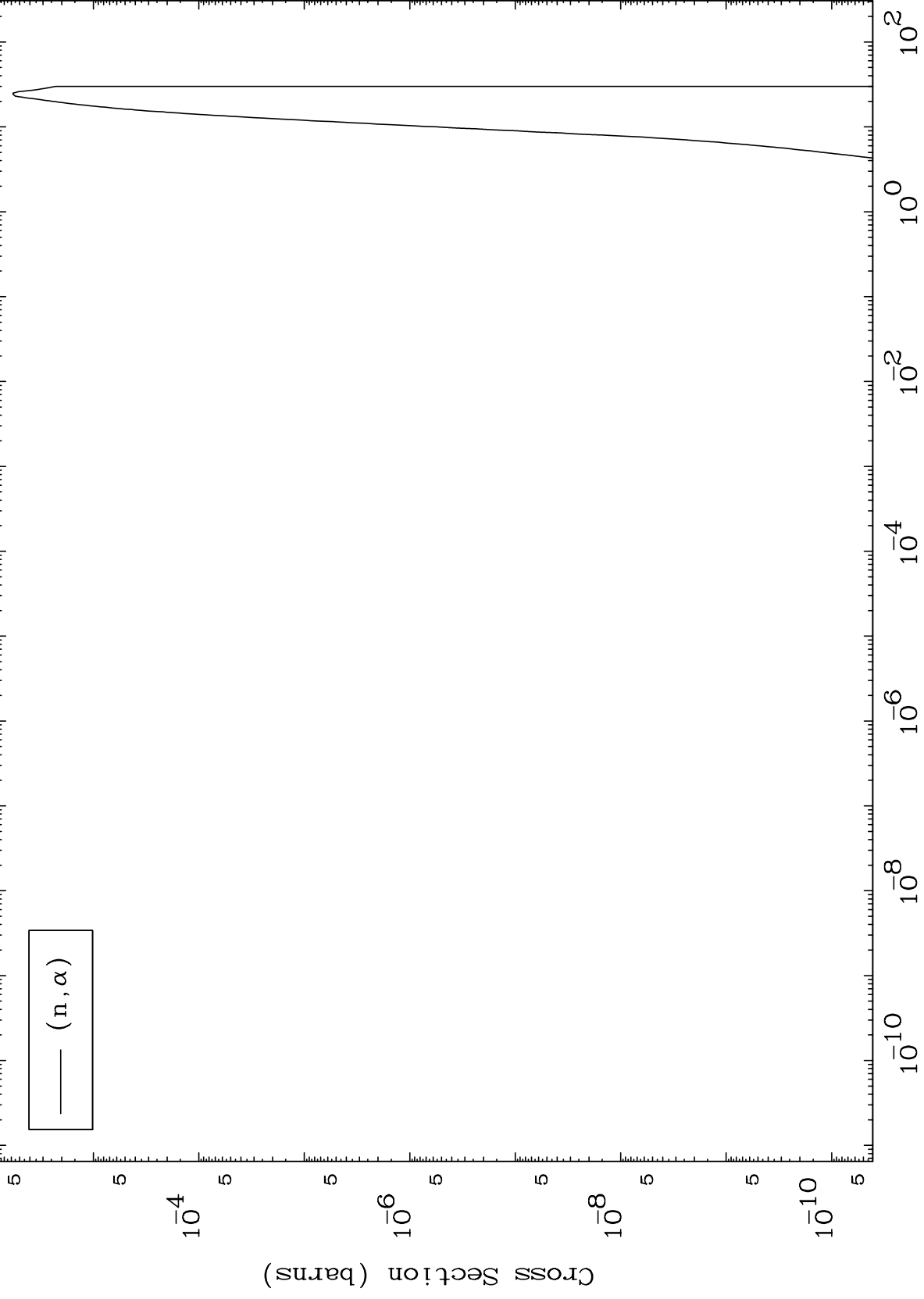
Incident Energy (MeV)

77-Ir-196m

MAT 7741

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

77-Ir-196m

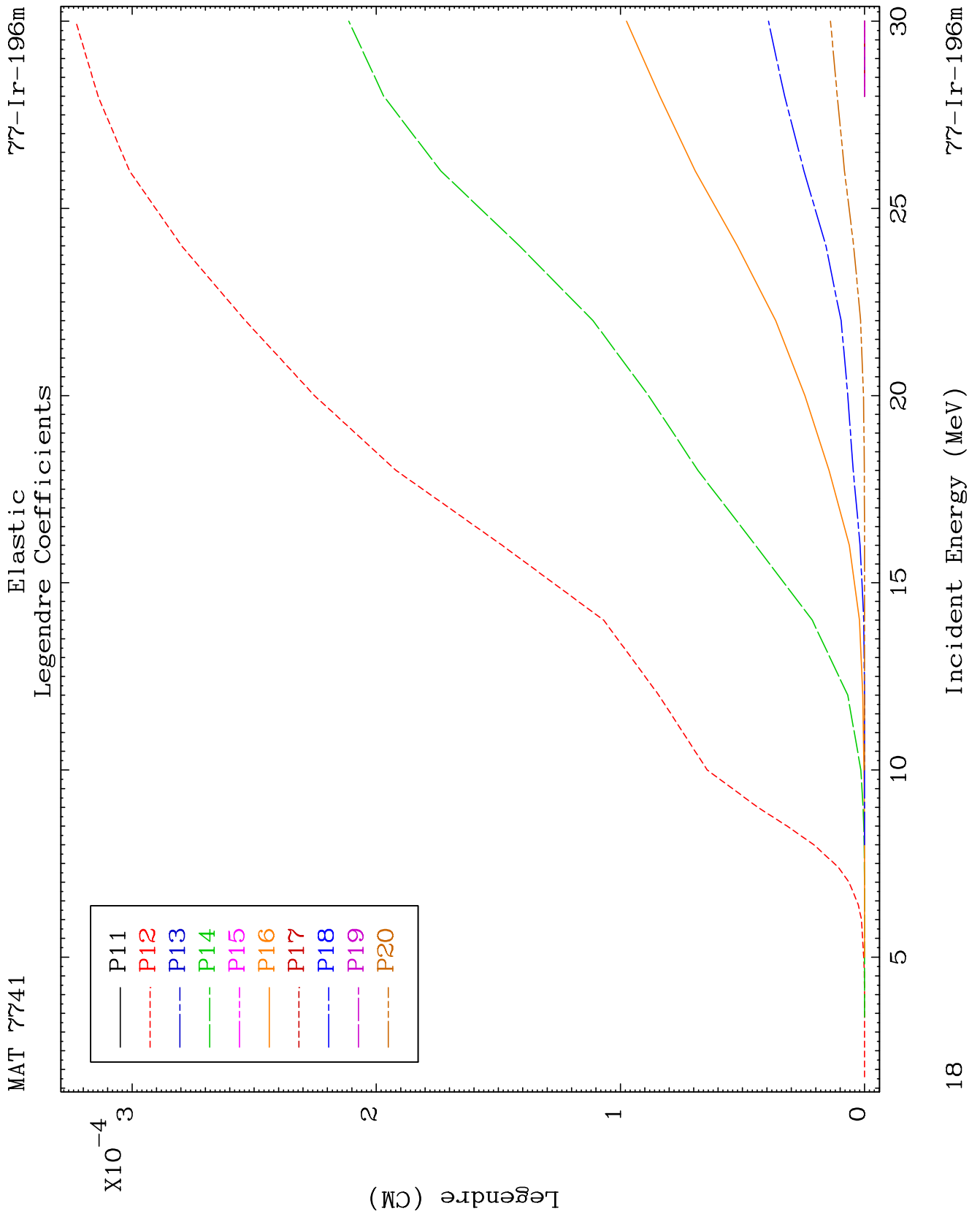


16

Incident Energy (MeV)

77-Ir-196m

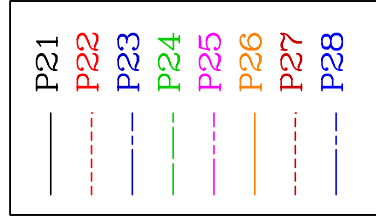




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Elastic  
Legendre Coefficients

77-Ir-196m



$\times 10^{-6}$   
2.0

1.5

Legendre (CM)

1.0

0.5

0.0

15

20

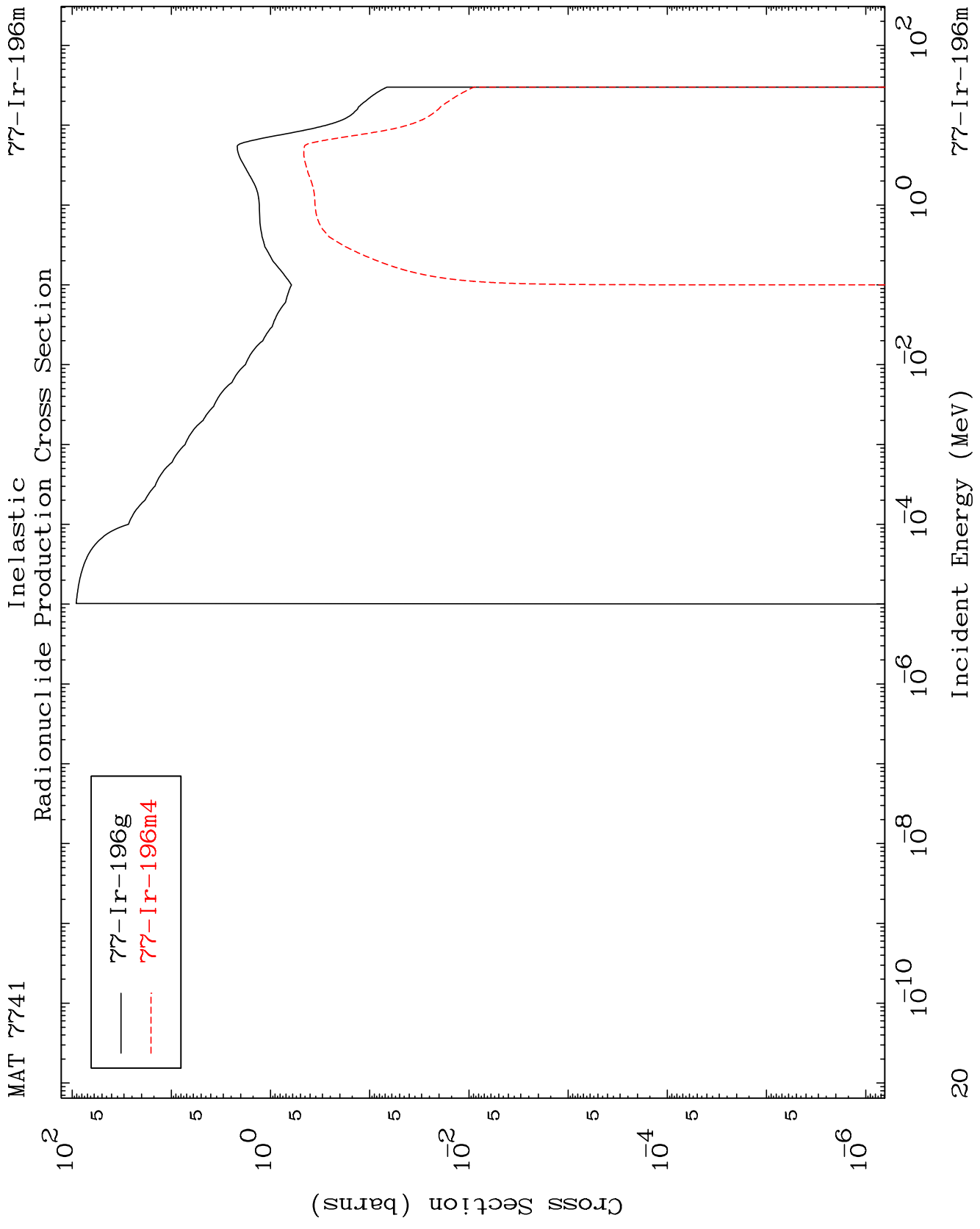
25

30

19

Incident Energy (MeV)

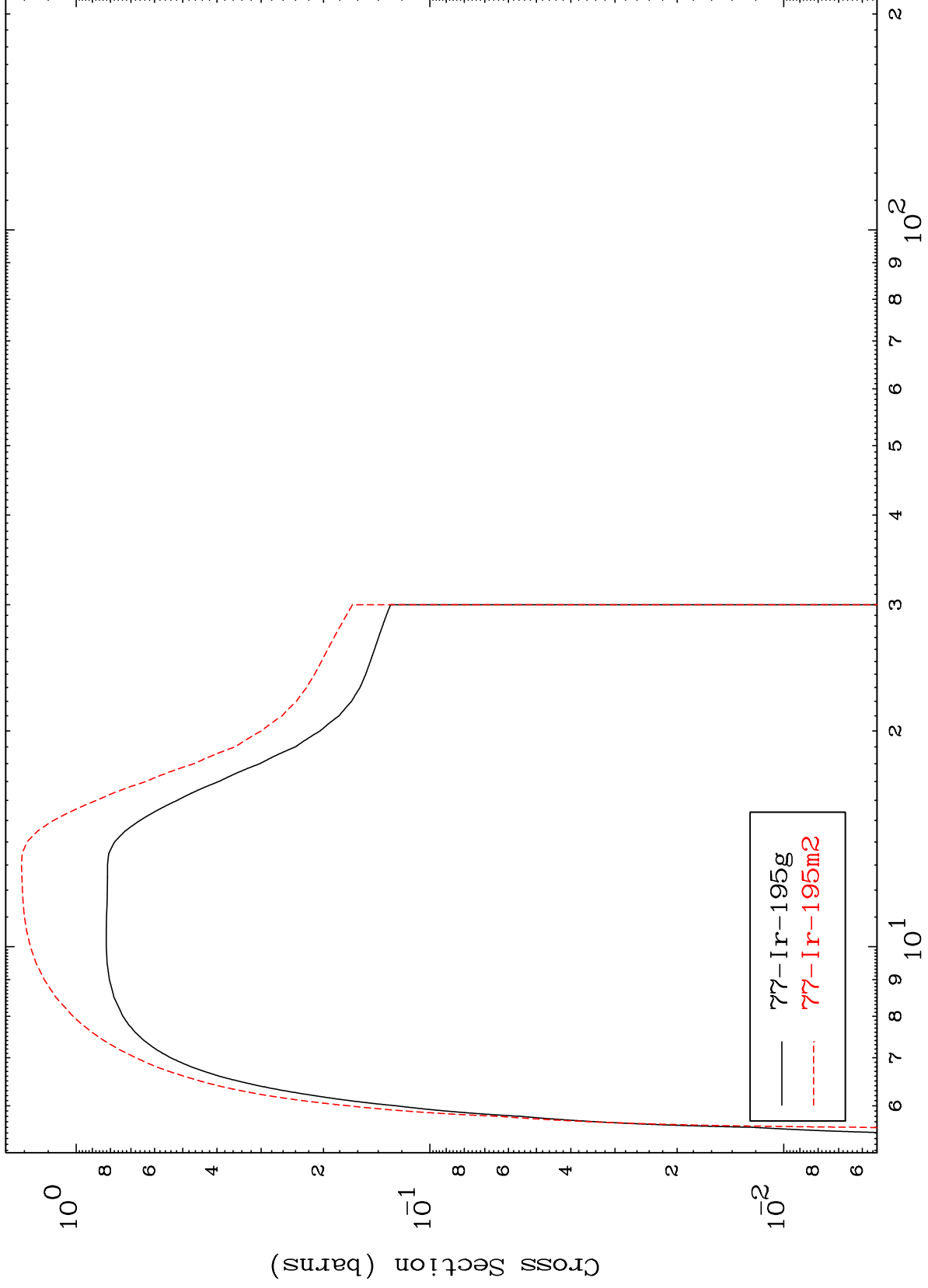
77-Ir-196m



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<sup>77</sup>Ir-196m

(n,2n)  
Radionuclide Production Cross Section



21

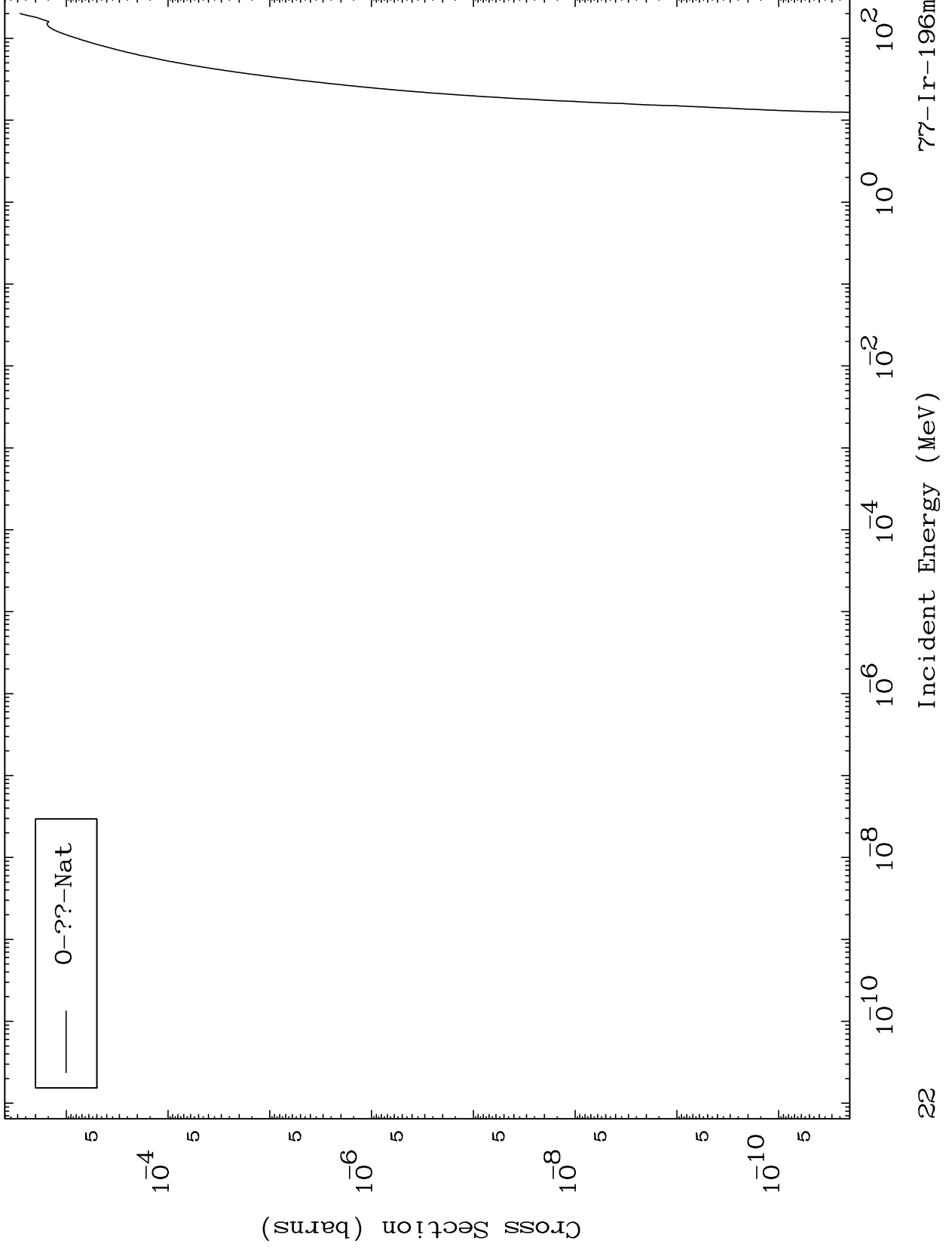
Incident Energy (MeV)

<sup>77</sup>Ir-196m

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Fission  
Radionuclide Production Cross Section

<sup>77</sup>Ir-196m

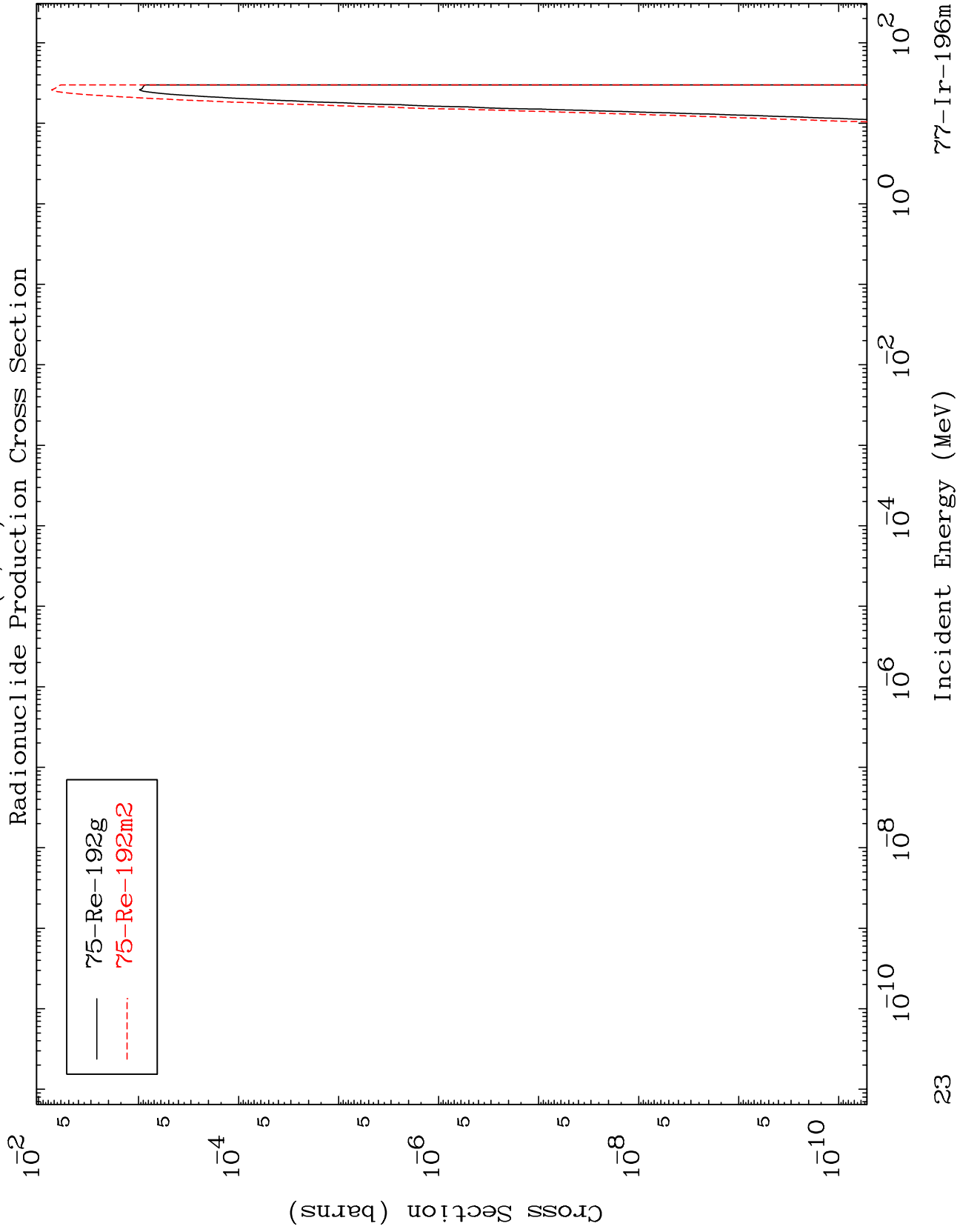


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$(n, n') \alpha$

$^{77}\text{Ir}-196\text{m}$

Radionuclide Production Cross Section

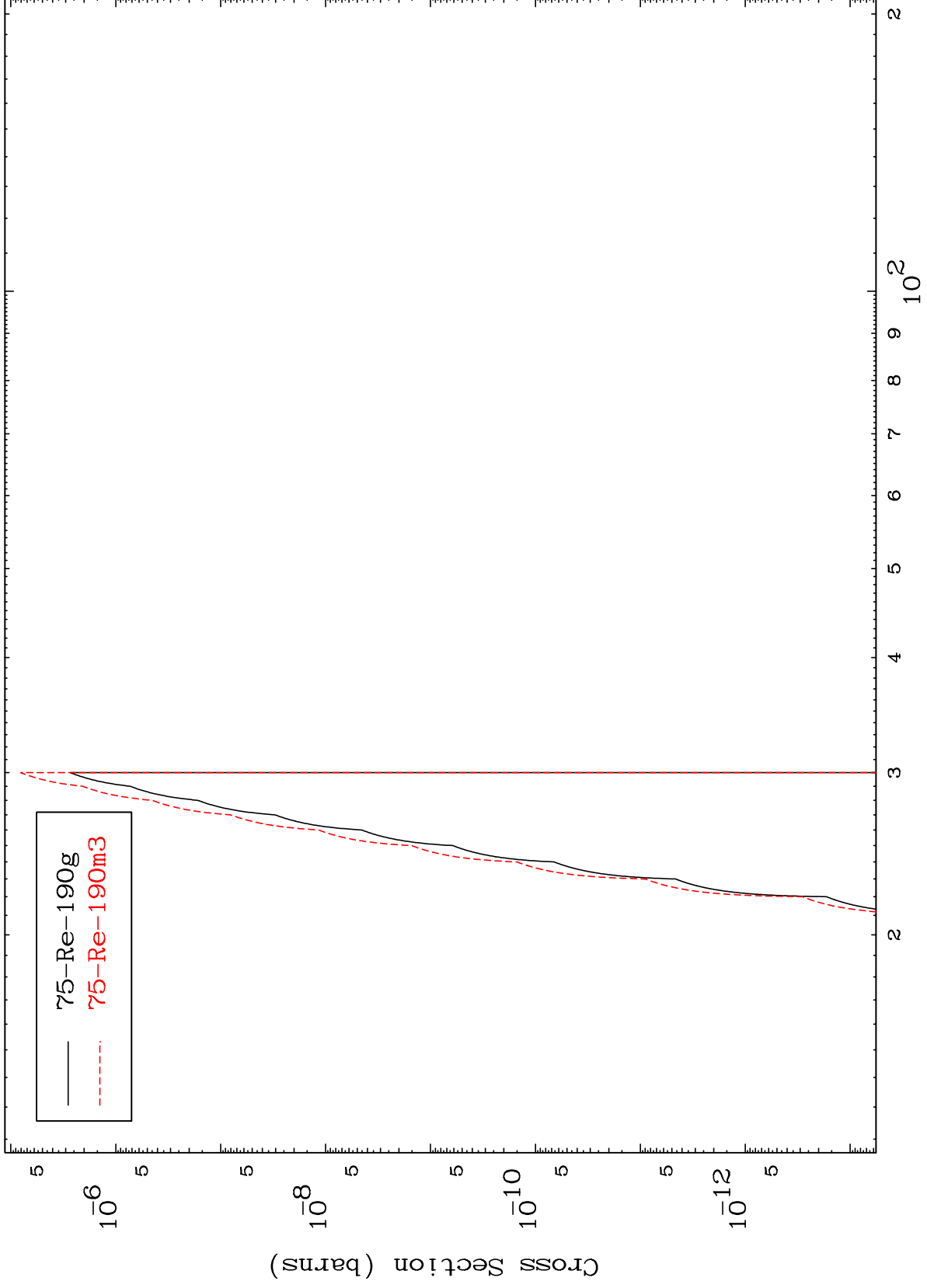


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(n,3n)  $\alpha$

77-Ir-196m

Radionuclide Production Cross Section



24

Incident Energy (MeV)

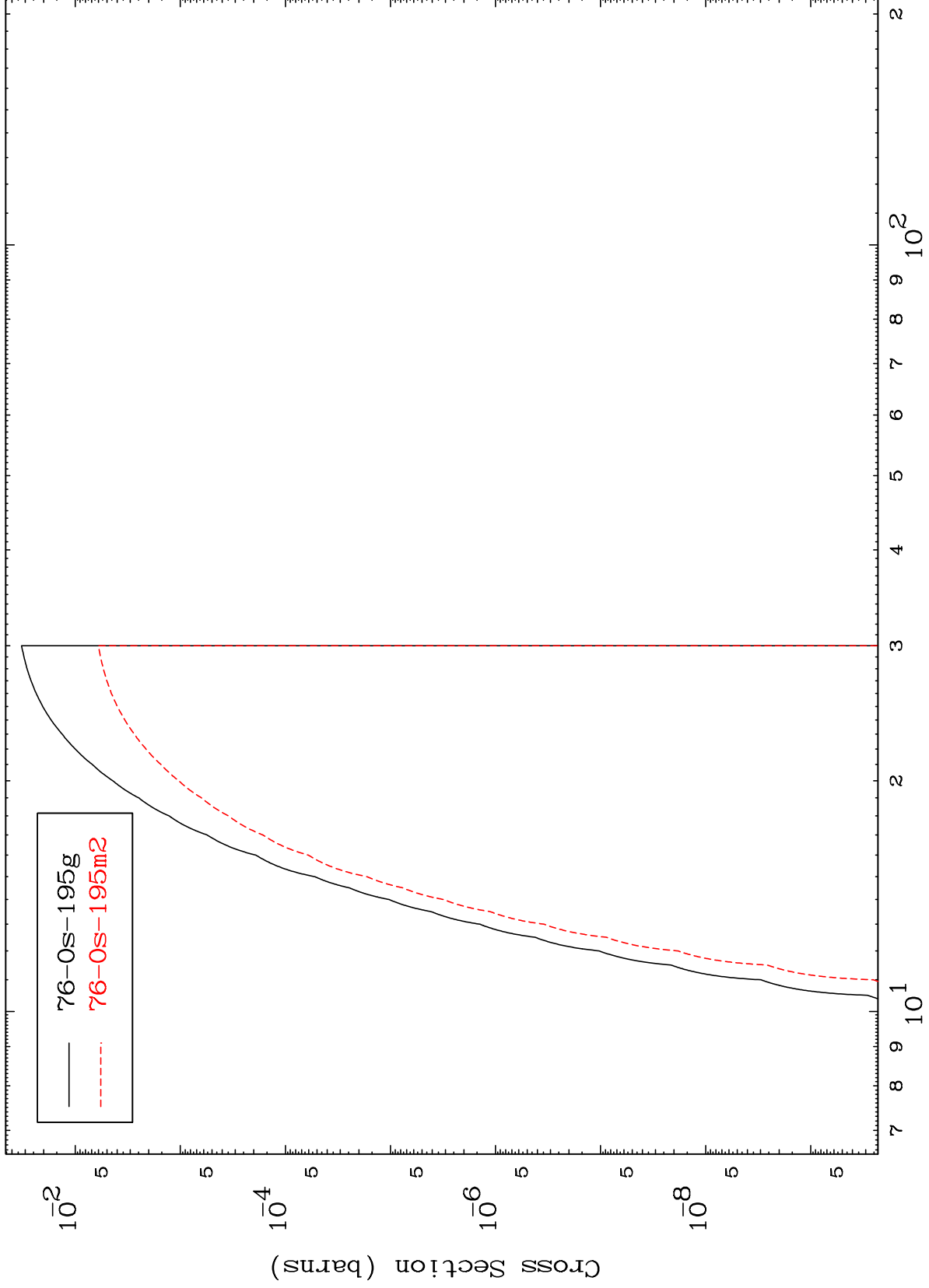
77-Ir-196m

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(n,n') p

77-Ir-196m

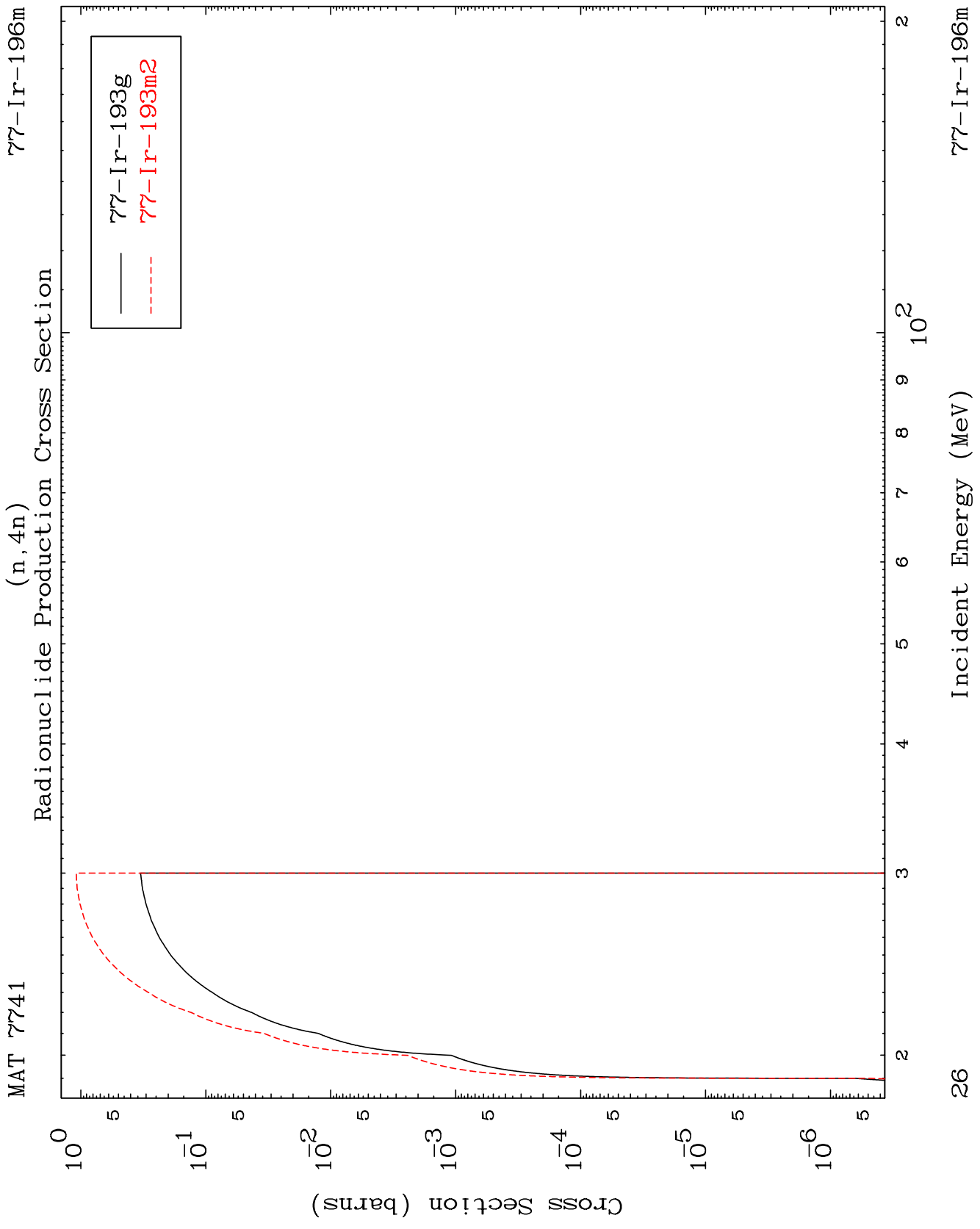
Radionuclide Production Cross Section



25

Incident Energy (MeV)

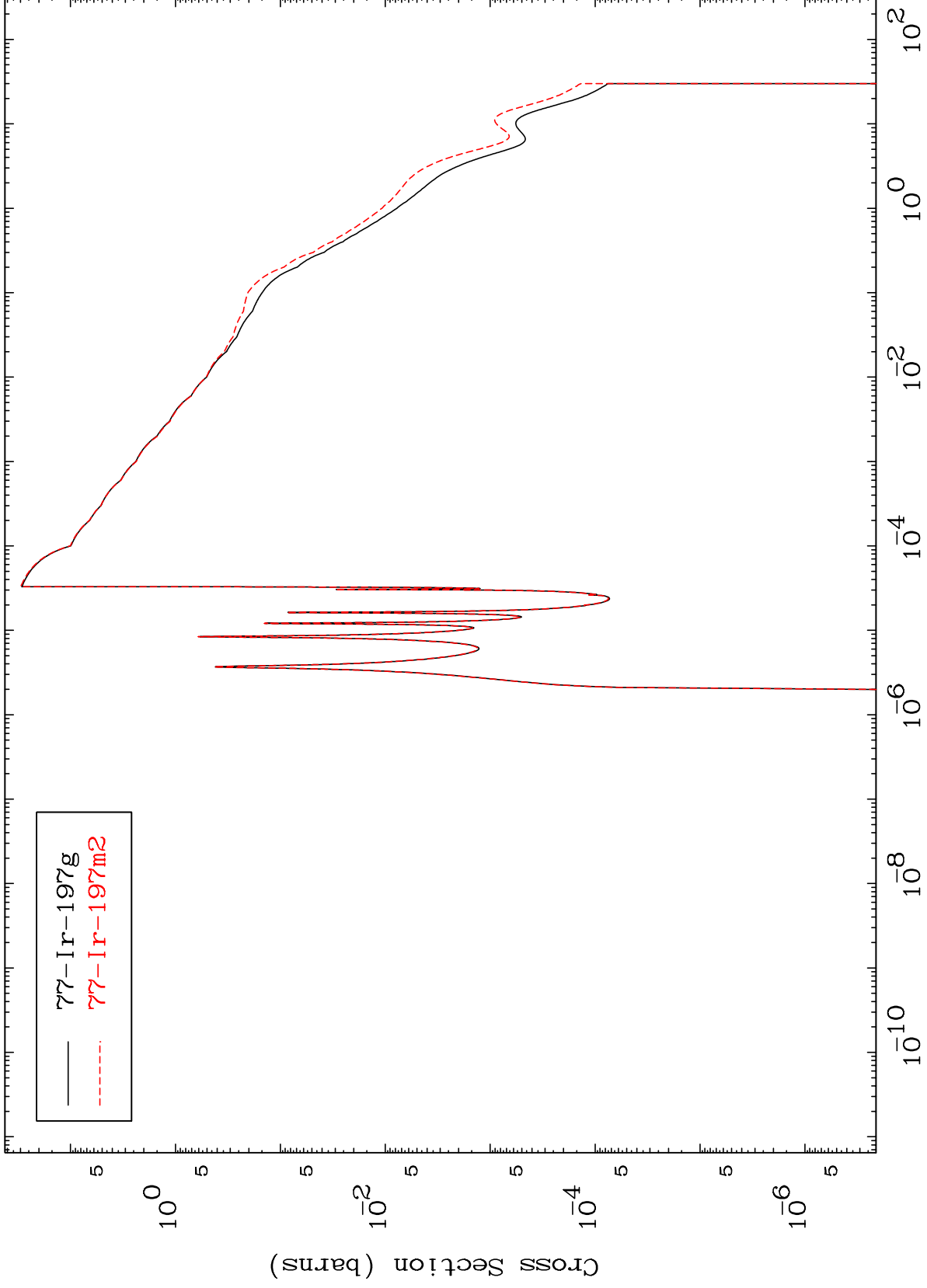
77-Ir-196m



MAT 7741

<sup>77</sup>Ir-196m

(n,γ)  
Radionuclide Production Cross Section



27

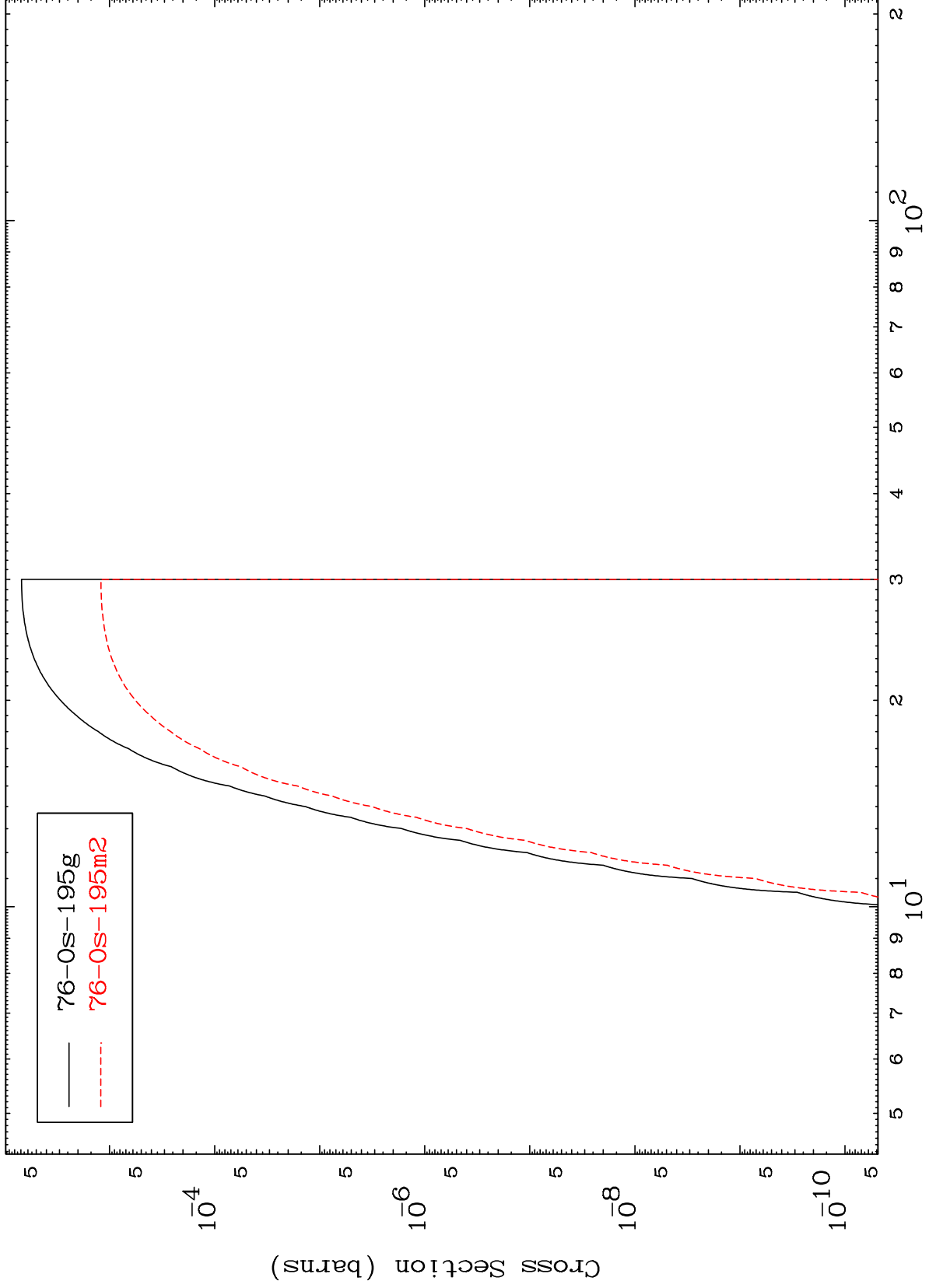
<sup>77</sup>Ir-196m

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(n,d)

<sup>77</sup>Ir-196m

Radionuclide Production Cross Section



28

Incident Energy (MeV)

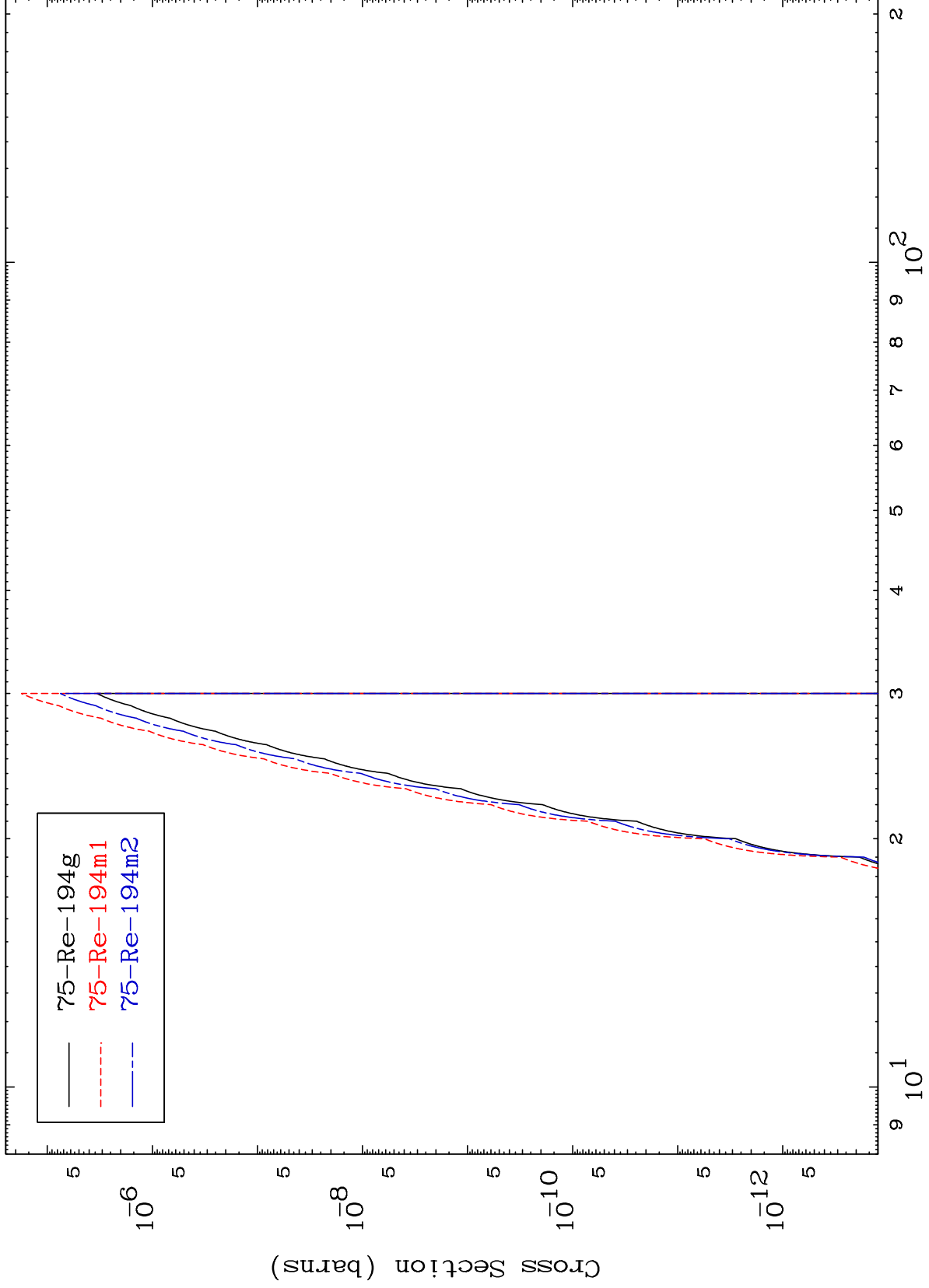
<sup>77</sup>Ir-196m

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(n,He-3)

77-Ir-196m

Radionuclide Production Cross Section



29

Incident Energy (MeV)

77-Ir-196m