

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

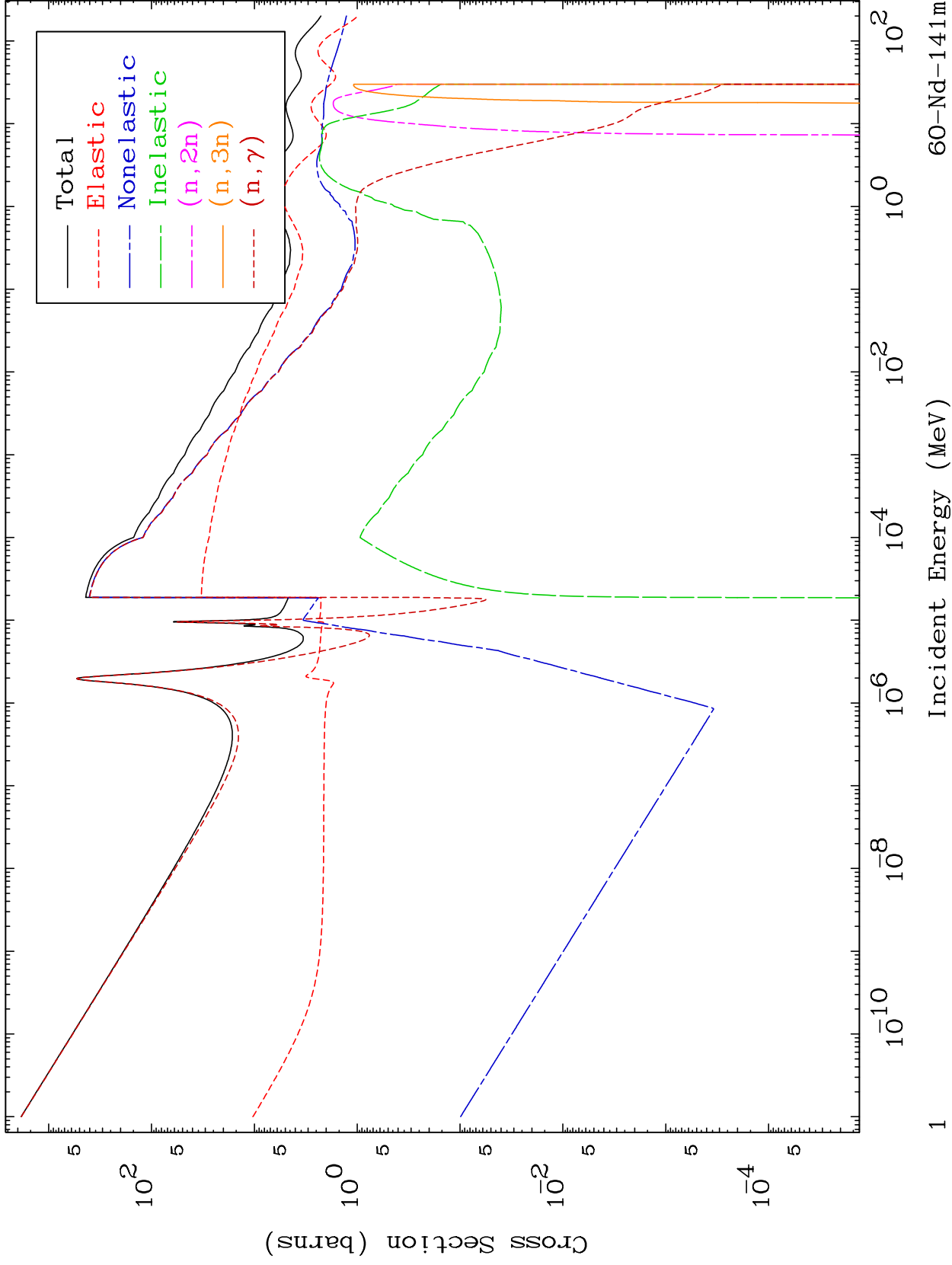
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 6023

Neutron Major  
293 Kelvin Cross Sections

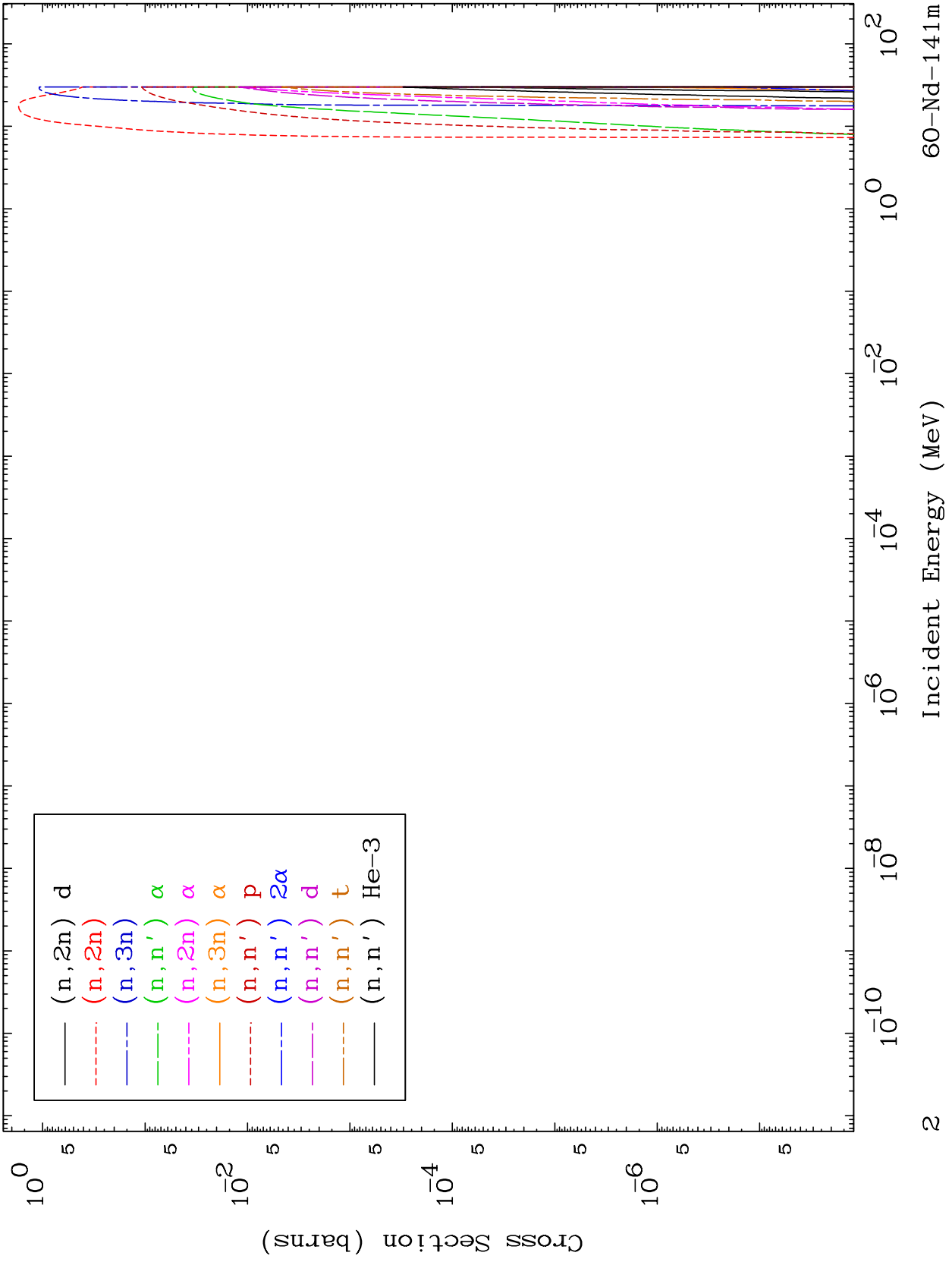
60-Nd-141m



MAT 6023

Neutron Absorption  
293 Kelvin Cross Sections

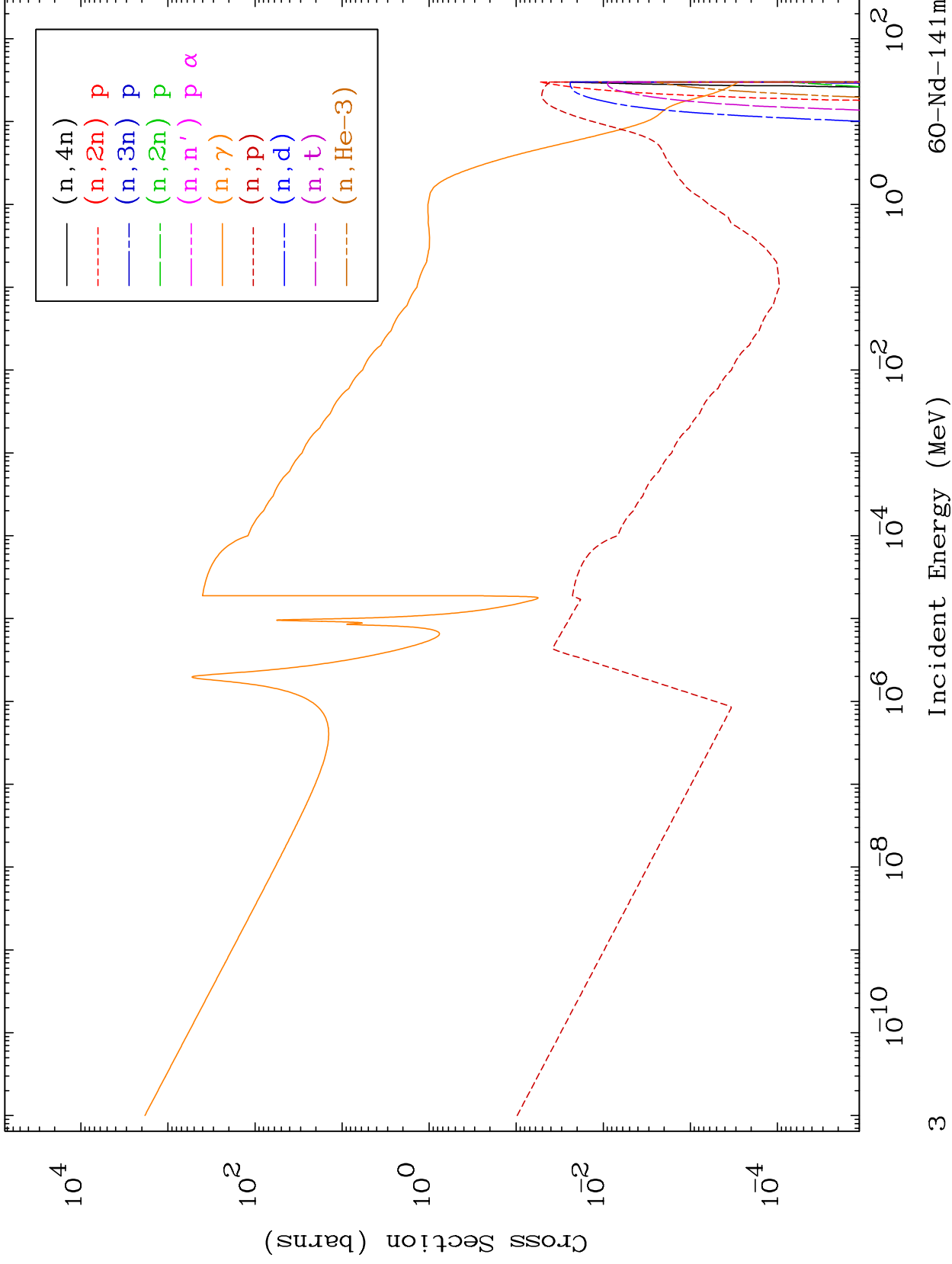
60-Nd-141m



MAT 6023

Neutron Absorption  
293 Kelvin Cross Sections

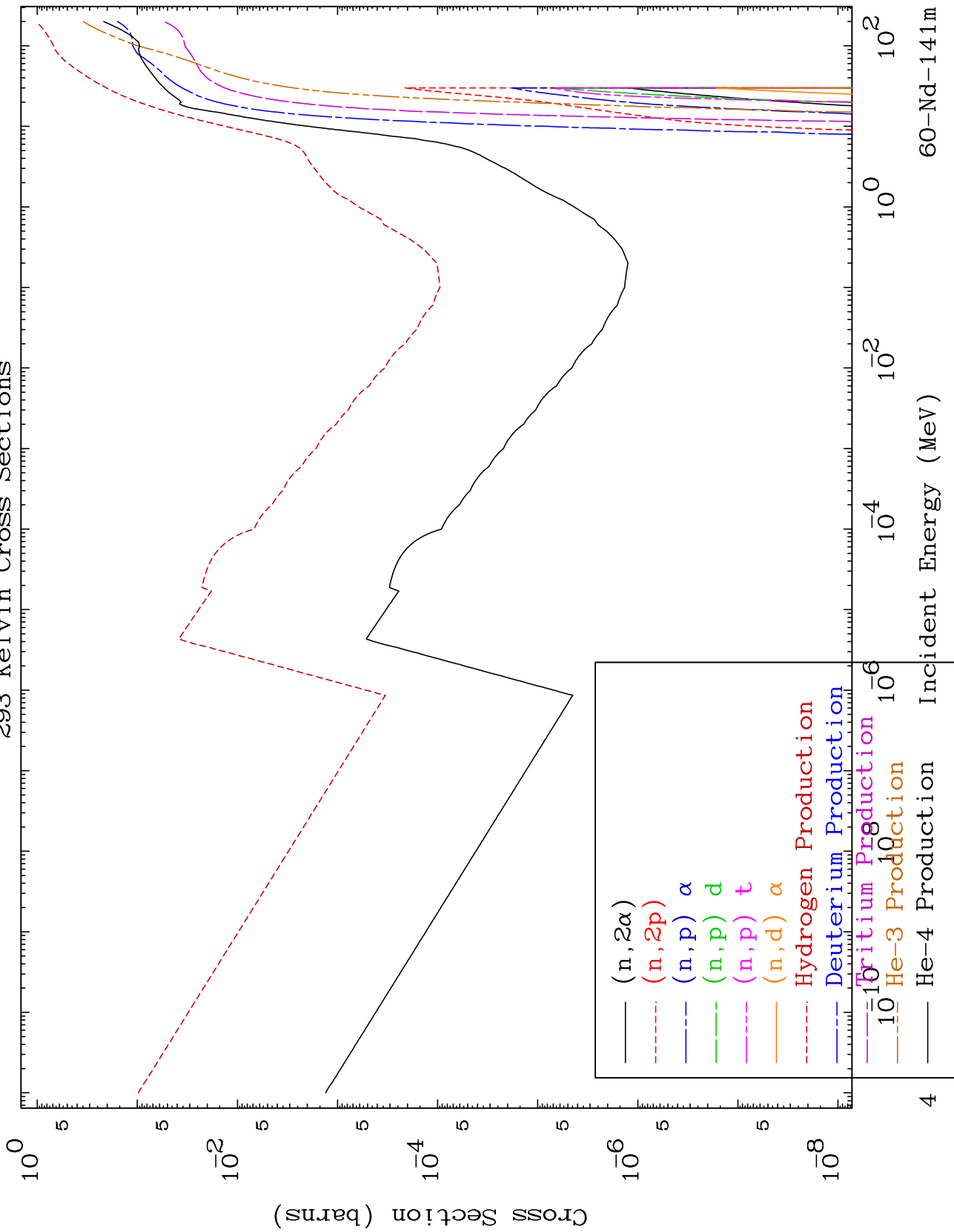
60-Nd-141m



MAT 6023

Neutron Absorption  
293 Kelvin Cross Sections

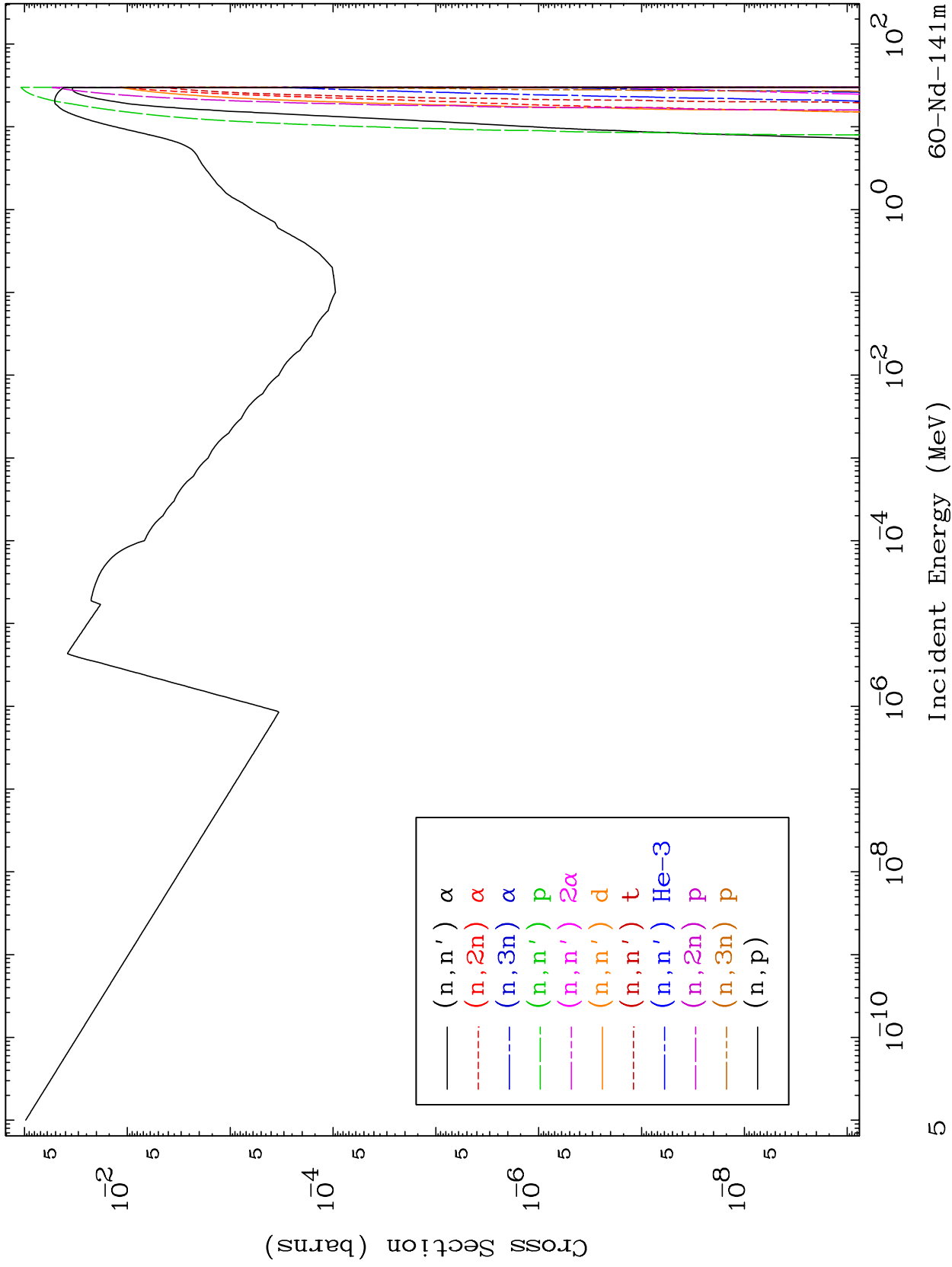
60-Nd-141m



MAT 6023

Charged Particle  
293 Kelvin Cross Sections

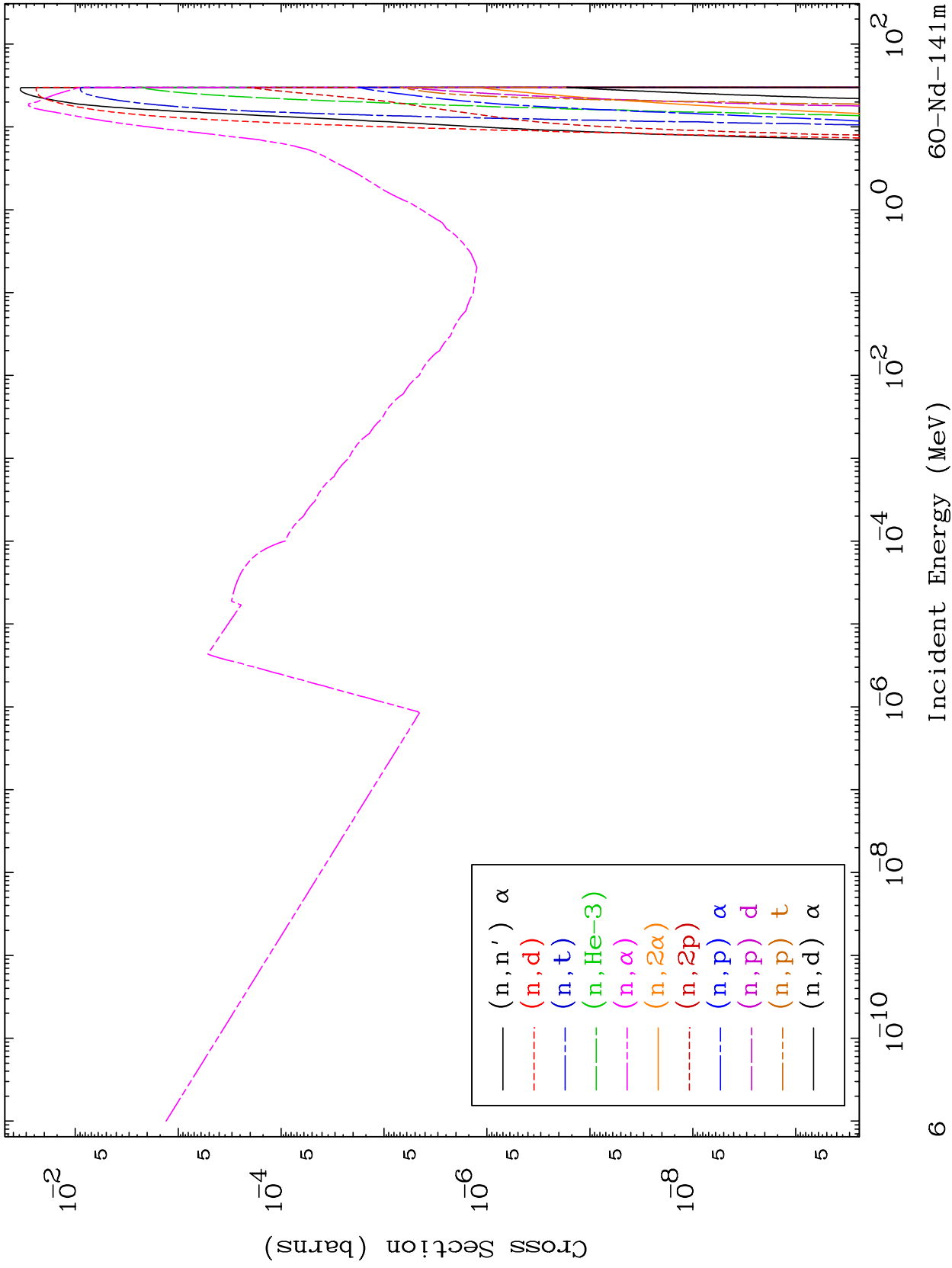
60-Nd-141m



MAT 6023

Charged Particle  
293 Kelvin Cross Sections

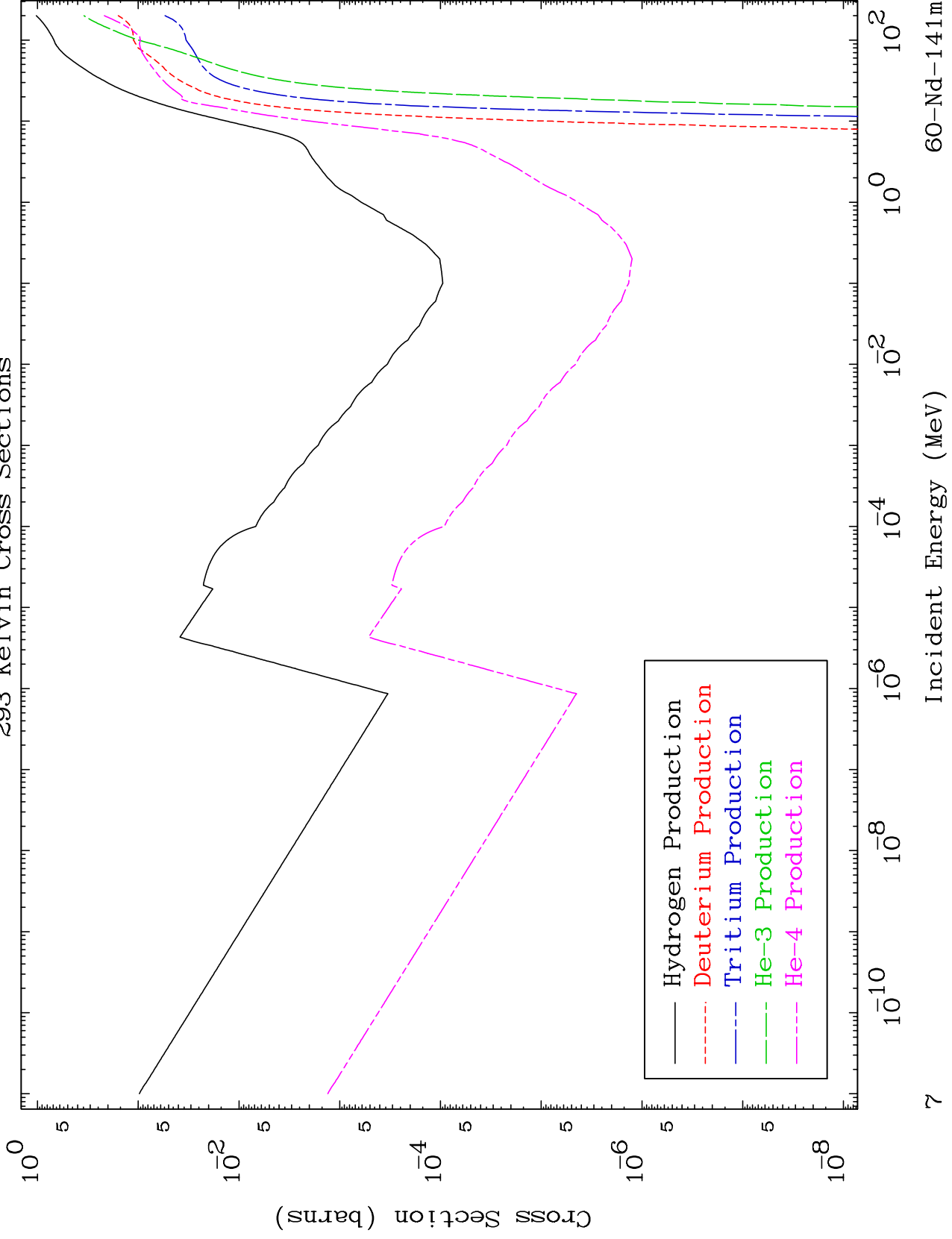
60-Nd-141m



MAT 6023

Particle Production  
293 Kelvin Cross Sections

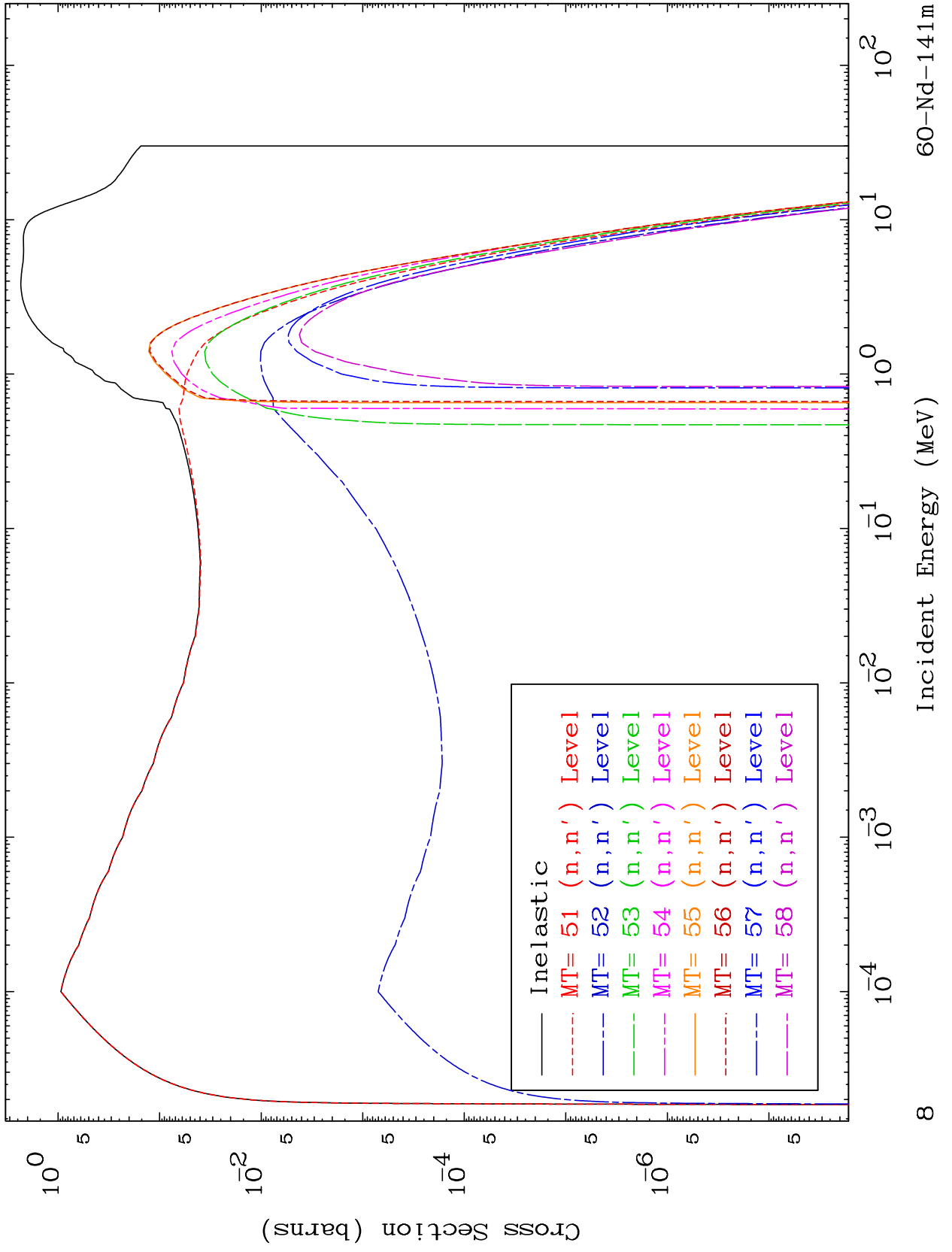
60-Nd-141m



MAT 6023

(n,n') Levels  
293 Kelvin Cross Sections

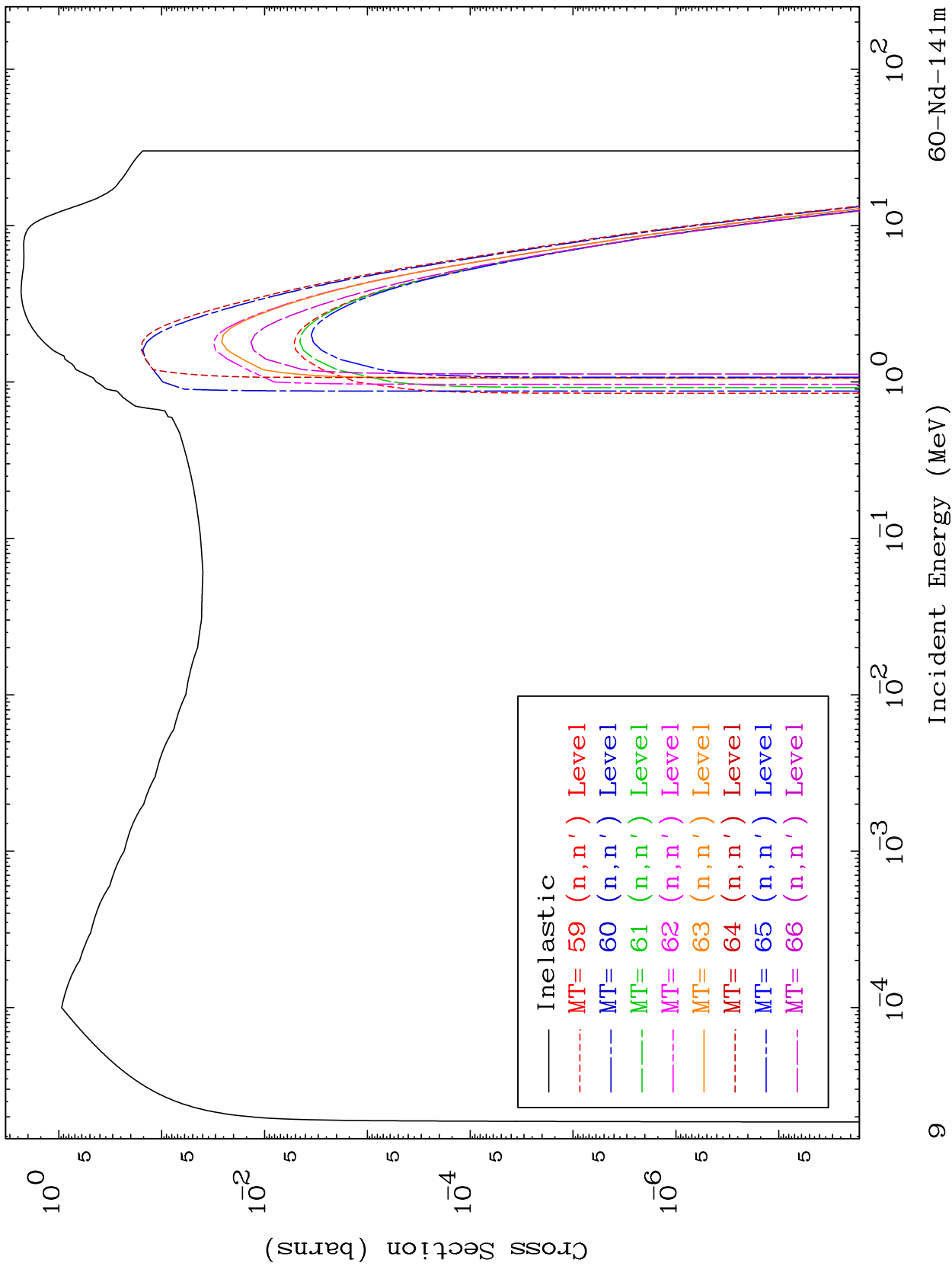
60-Nd-141m



MAT 6023

(n,n') Levels  
293 Kelvin Cross Sections

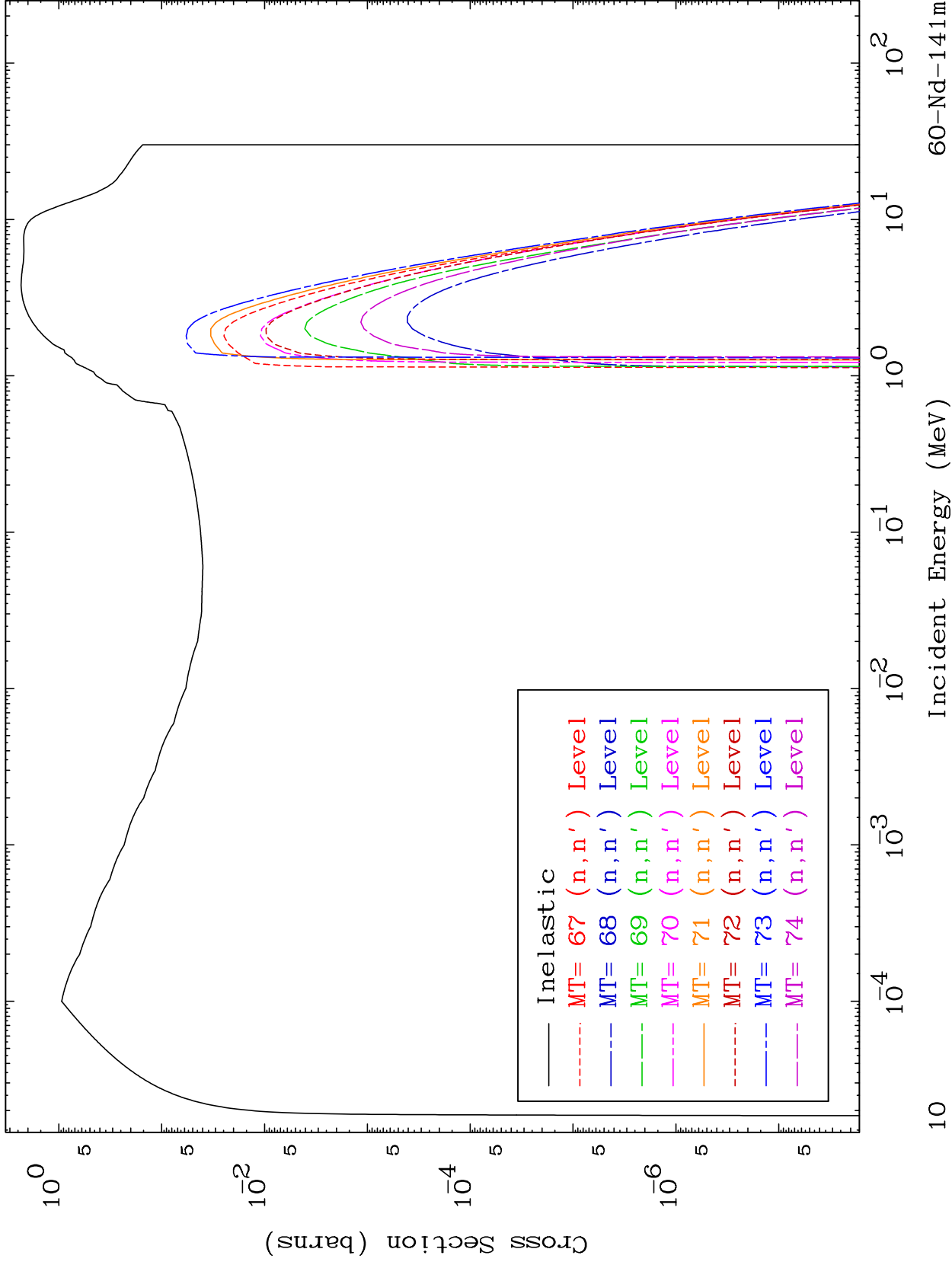
60-Nd-141m



MAT 6023

(n,n') Levels  
293 Kelvin Cross Sections

60-Nd-141m



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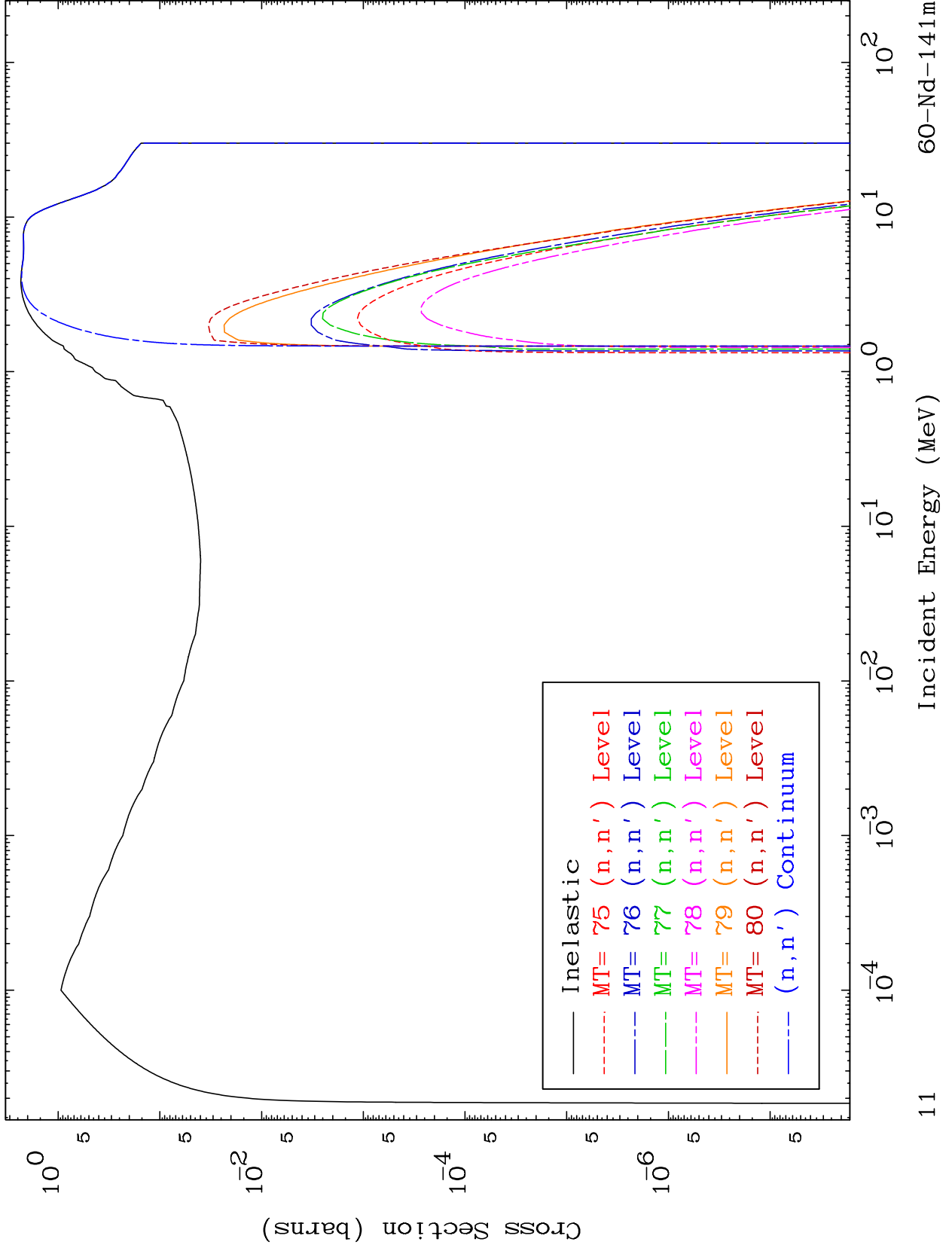
Incident Energy (MeV)

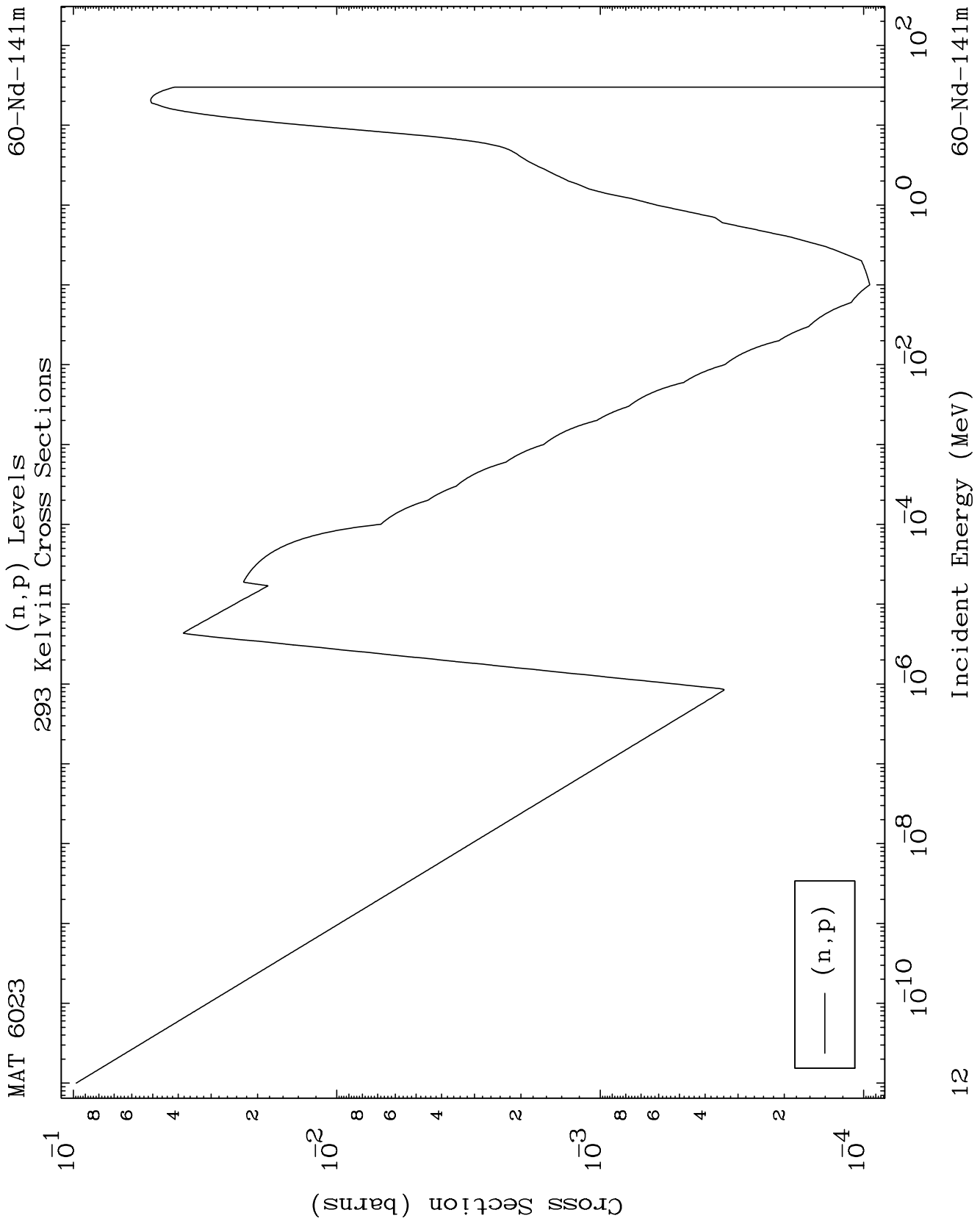
60-Nd-141m

MAT 6023

(n,n') Levels  
293 Kelvin Cross Sections

60-Nd-141m

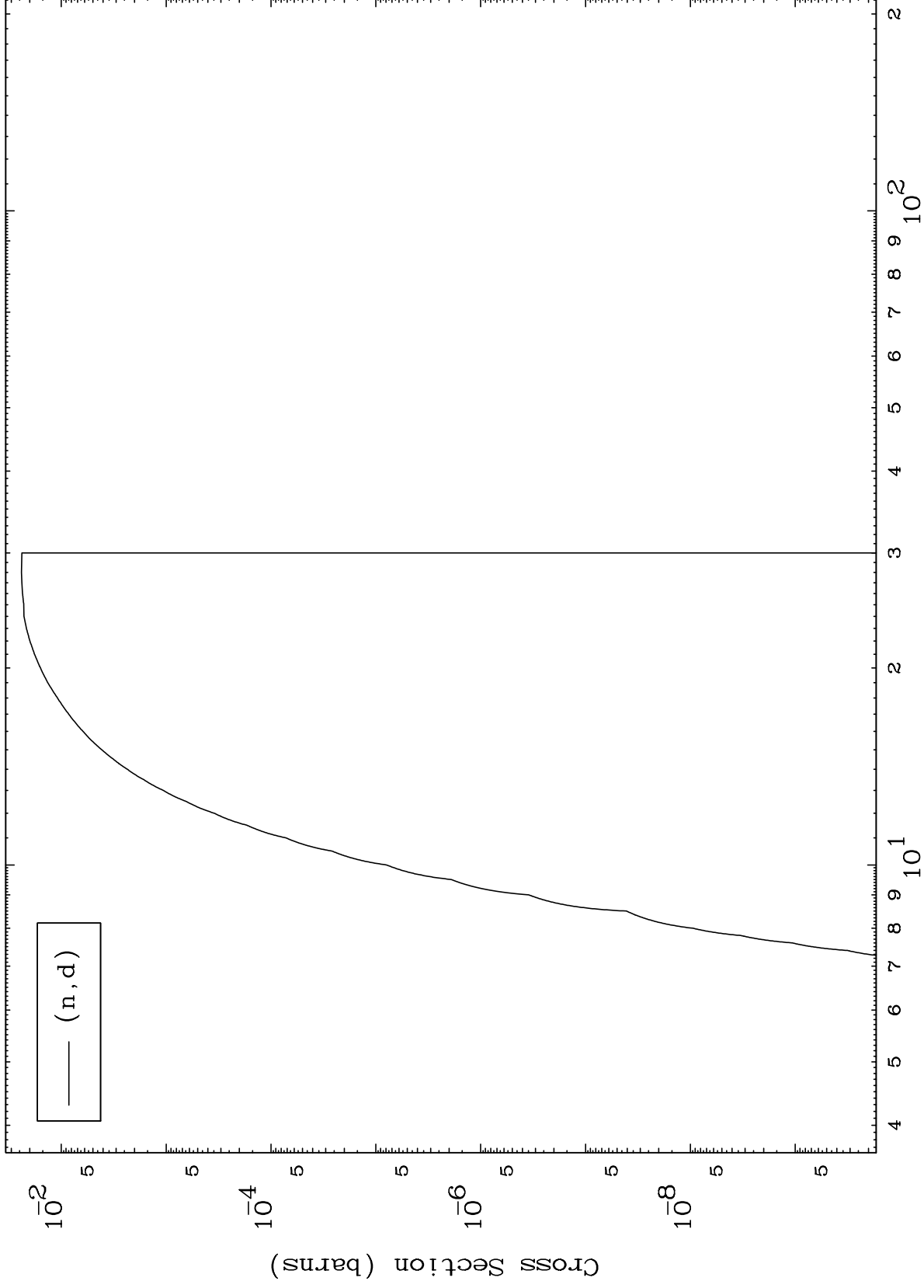




MAT 6023

(n,d) Levels  
293 Kelvin Cross Sections

60-Nd-141m



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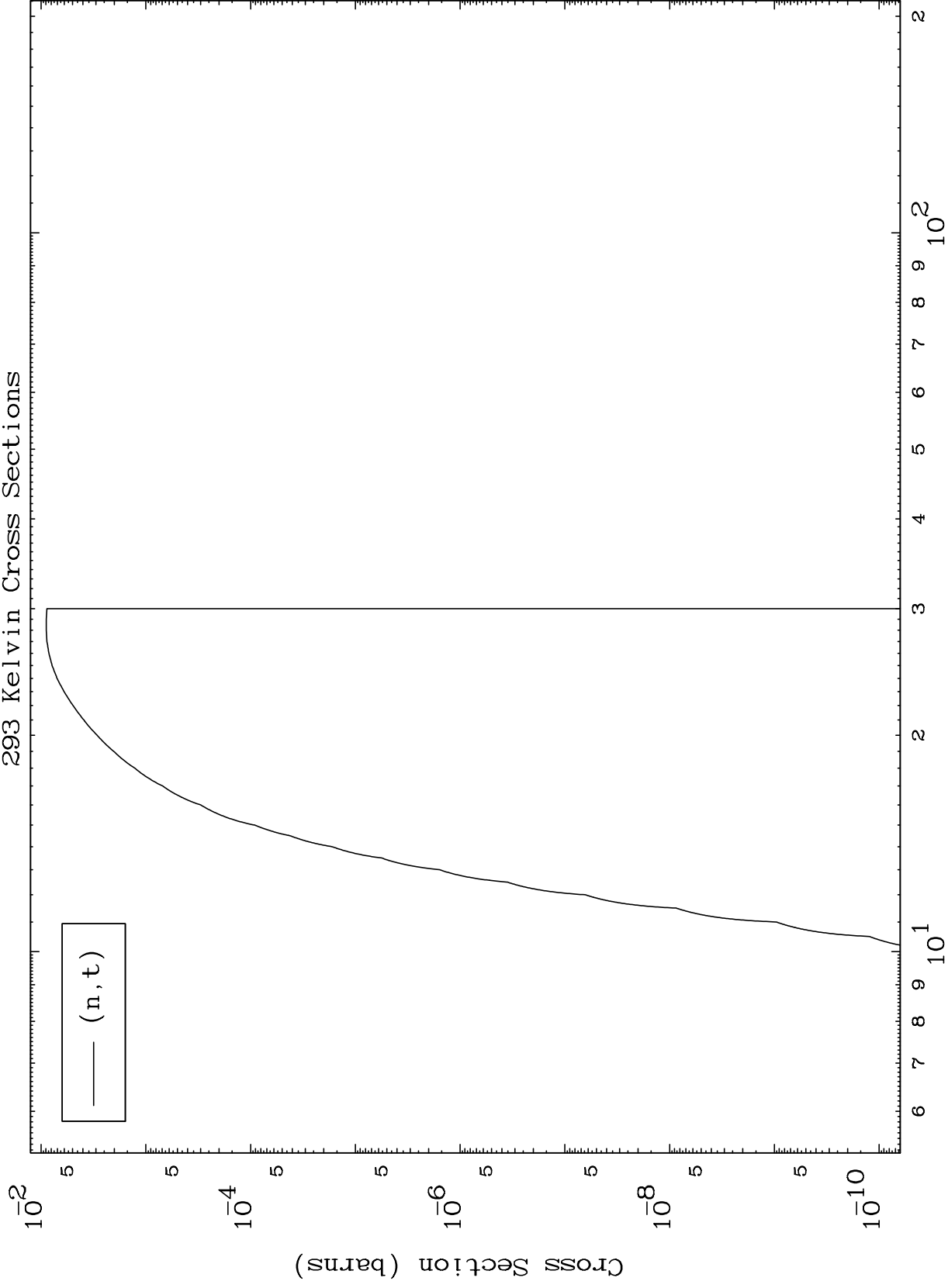
Incident Energy (MeV)

60-Nd-141m

MAT 6023

(n, t) Levels  
293 Kelvin Cross Sections

60-Nd-141m



14

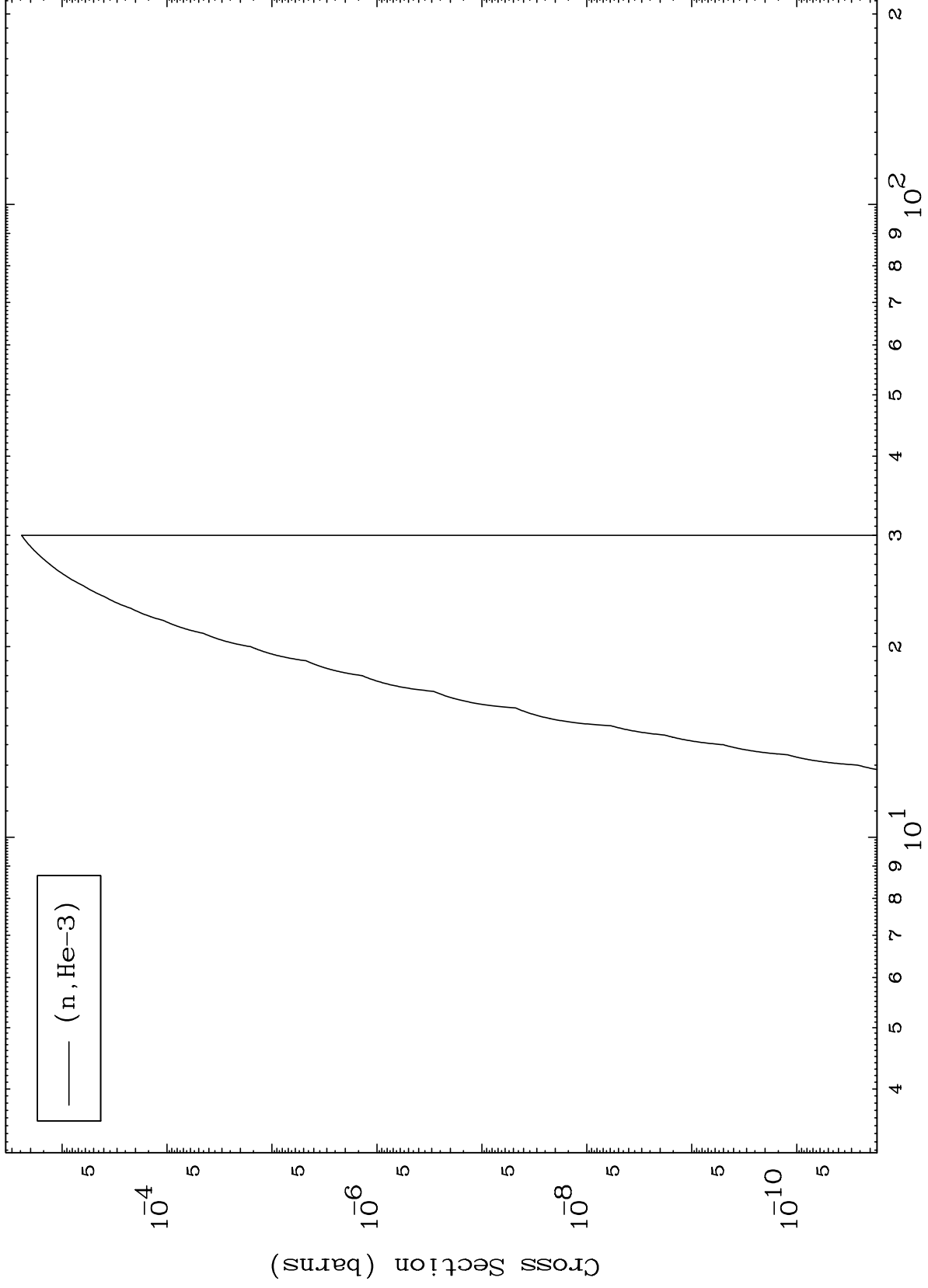
Incident Energy (MeV)

60-Nd-141m

MAT 6023

(n,He3) Levels  
293 Kelvin Cross Sections

60-Nd-141m



15

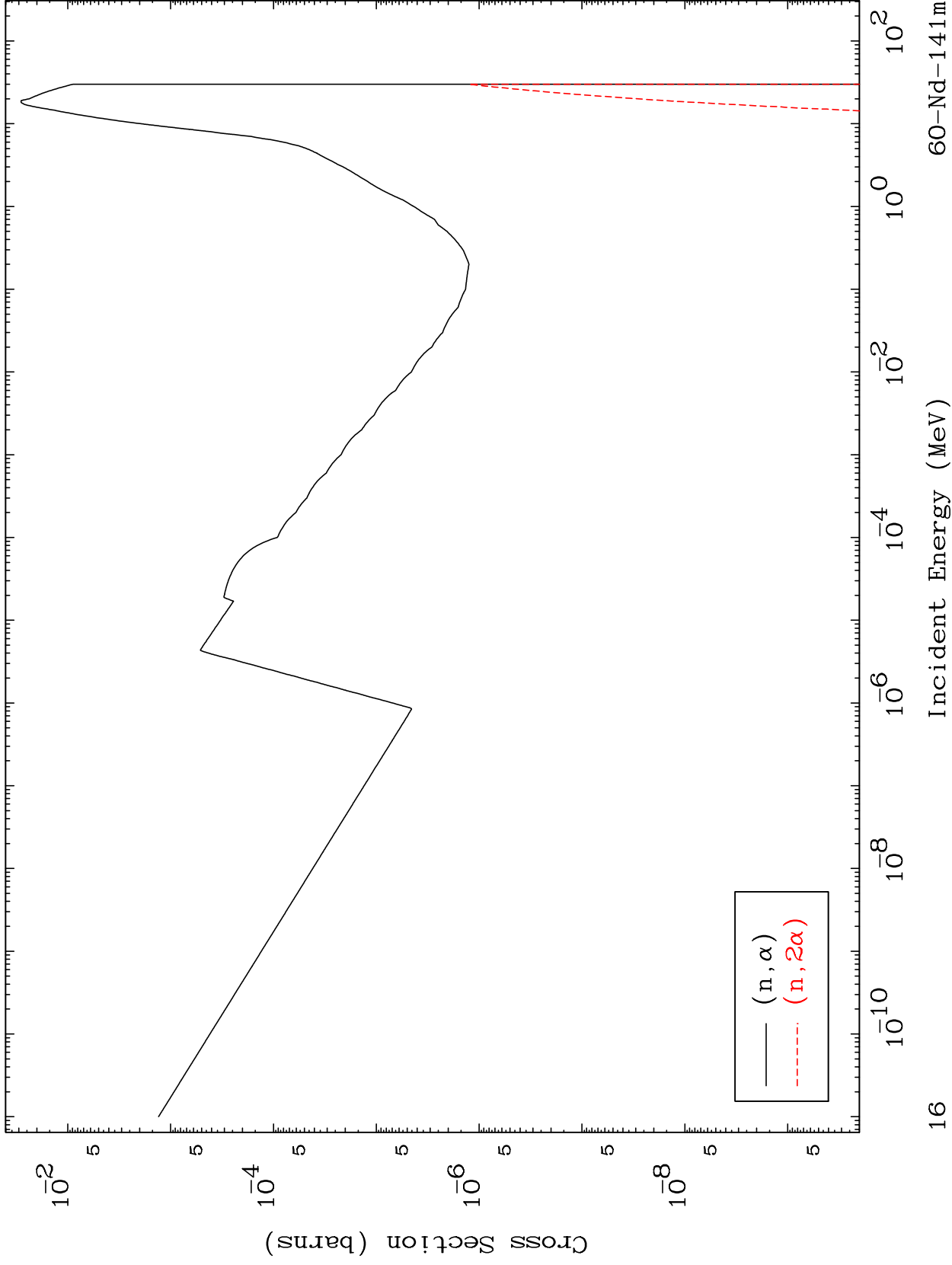
Incident Energy (MeV)

60-Nd-141m

MAT 6023

(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

60-Nd-141m

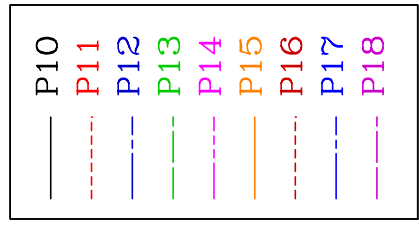




MAT 6023

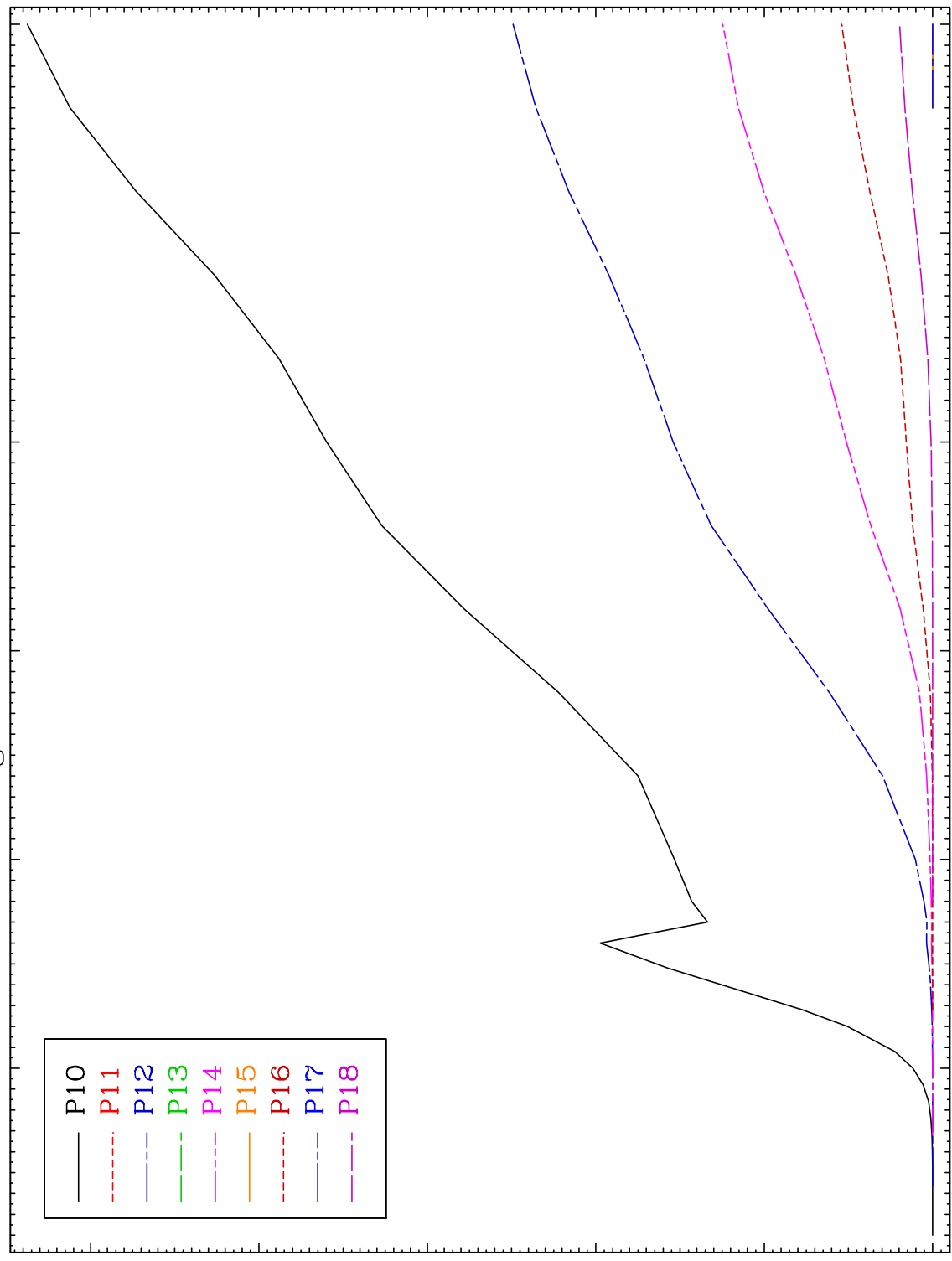
Elastic Legendre Coefficients

60-Nd-141m



$\times 10^{-4}$

Legendre (CM)



18

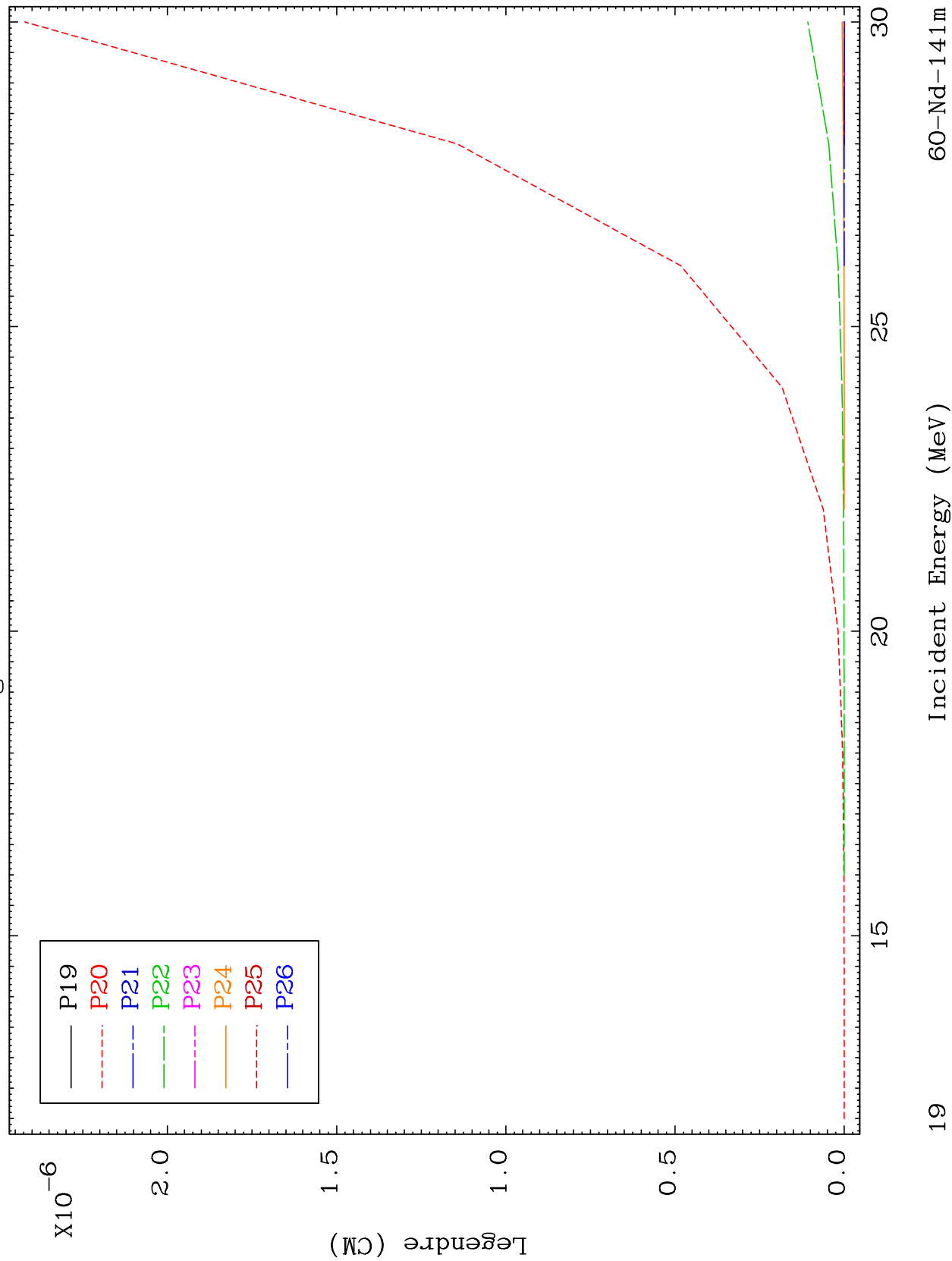
Incident Energy (MeV)

60-Nd-141m

MAT 6023

Elastic  
Legendre Coefficients

60-Nd-141m



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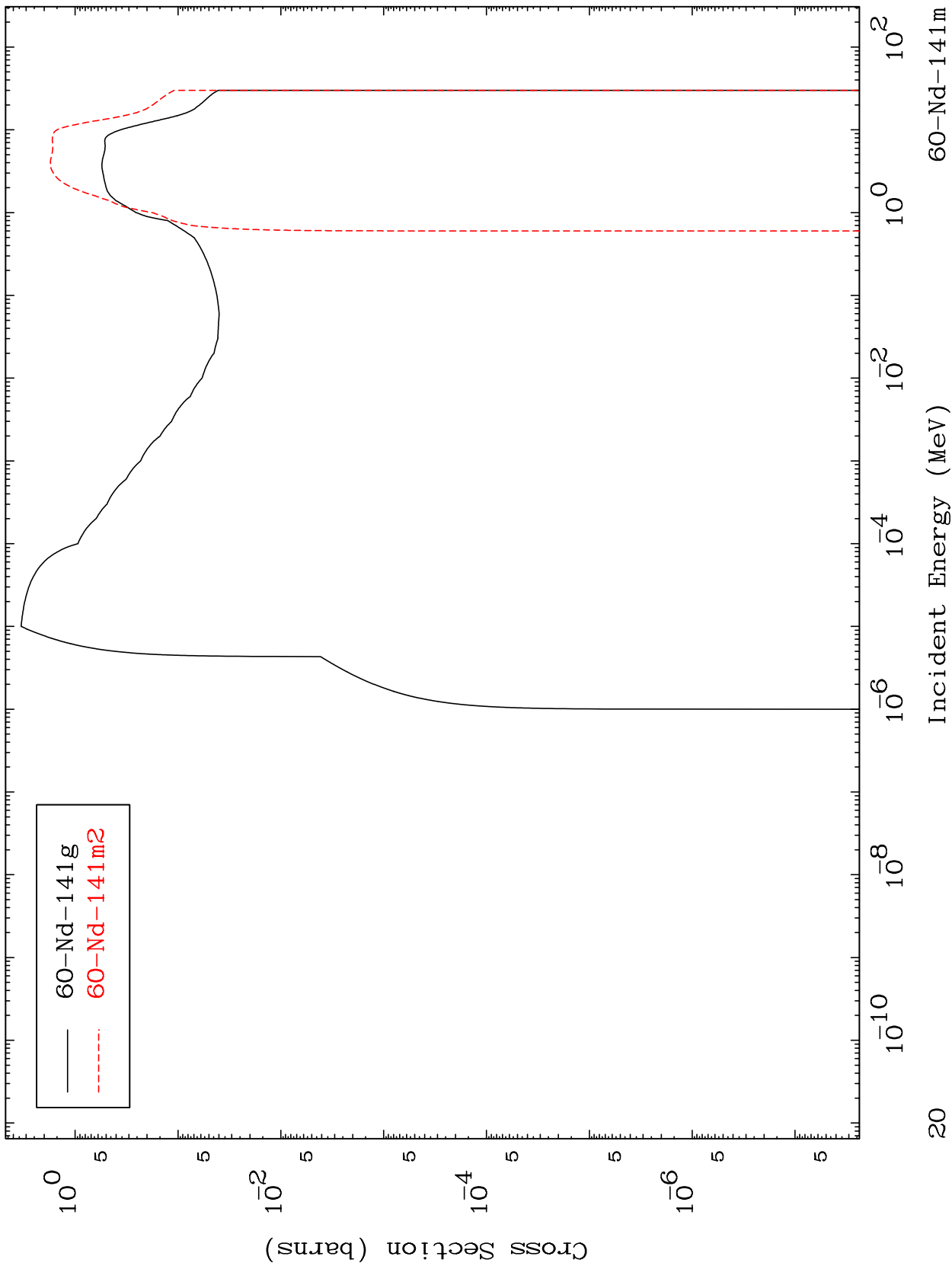
60-Nd-141m

Incident Energy (MeV)

MAT 6023

60-Nd-141m

### Inelastic Radionuclide Production Cross Section

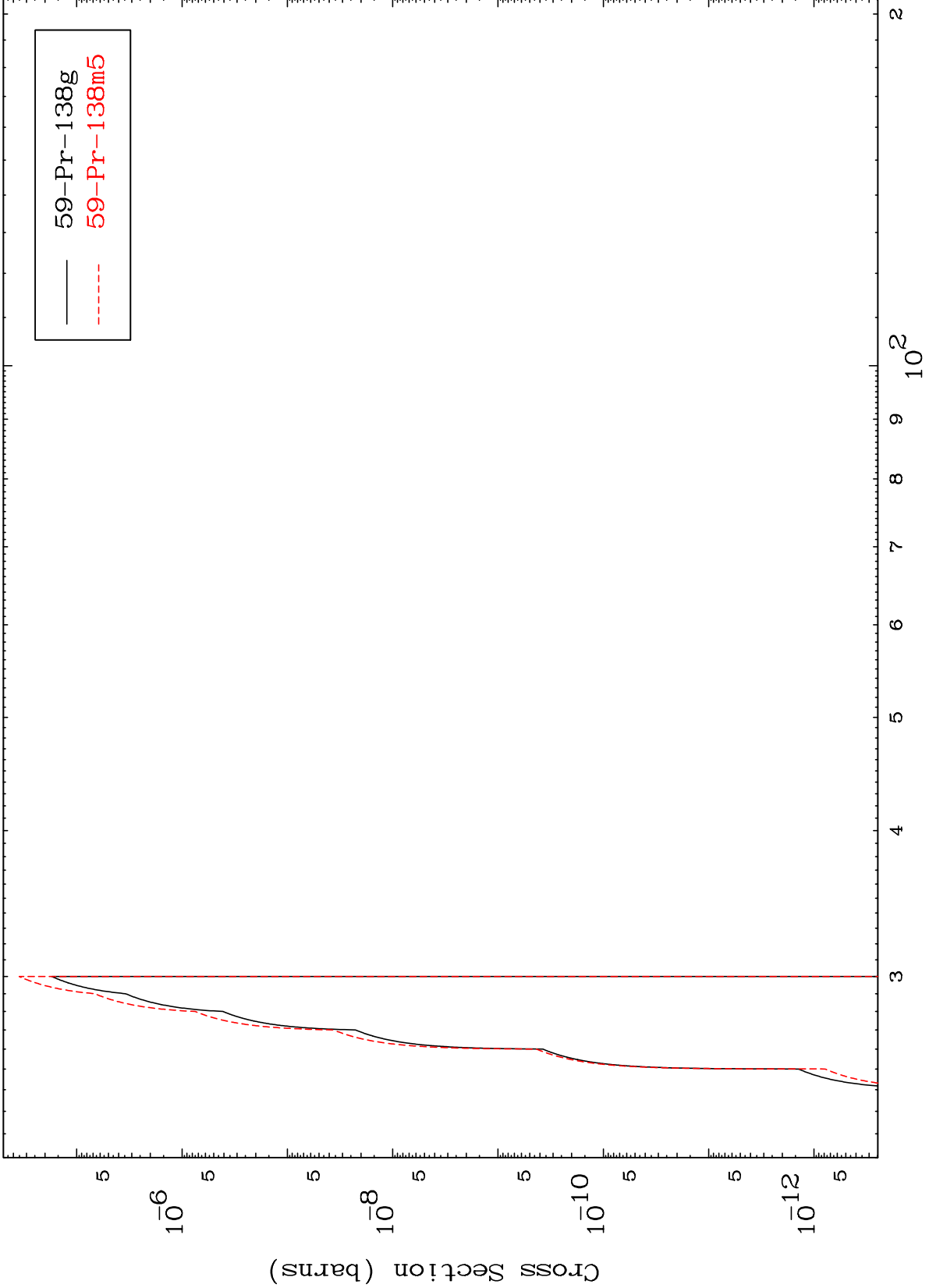


MAT 6023

(n,2n) d

60-Nd-141m

Radionuclide Production Cross Section

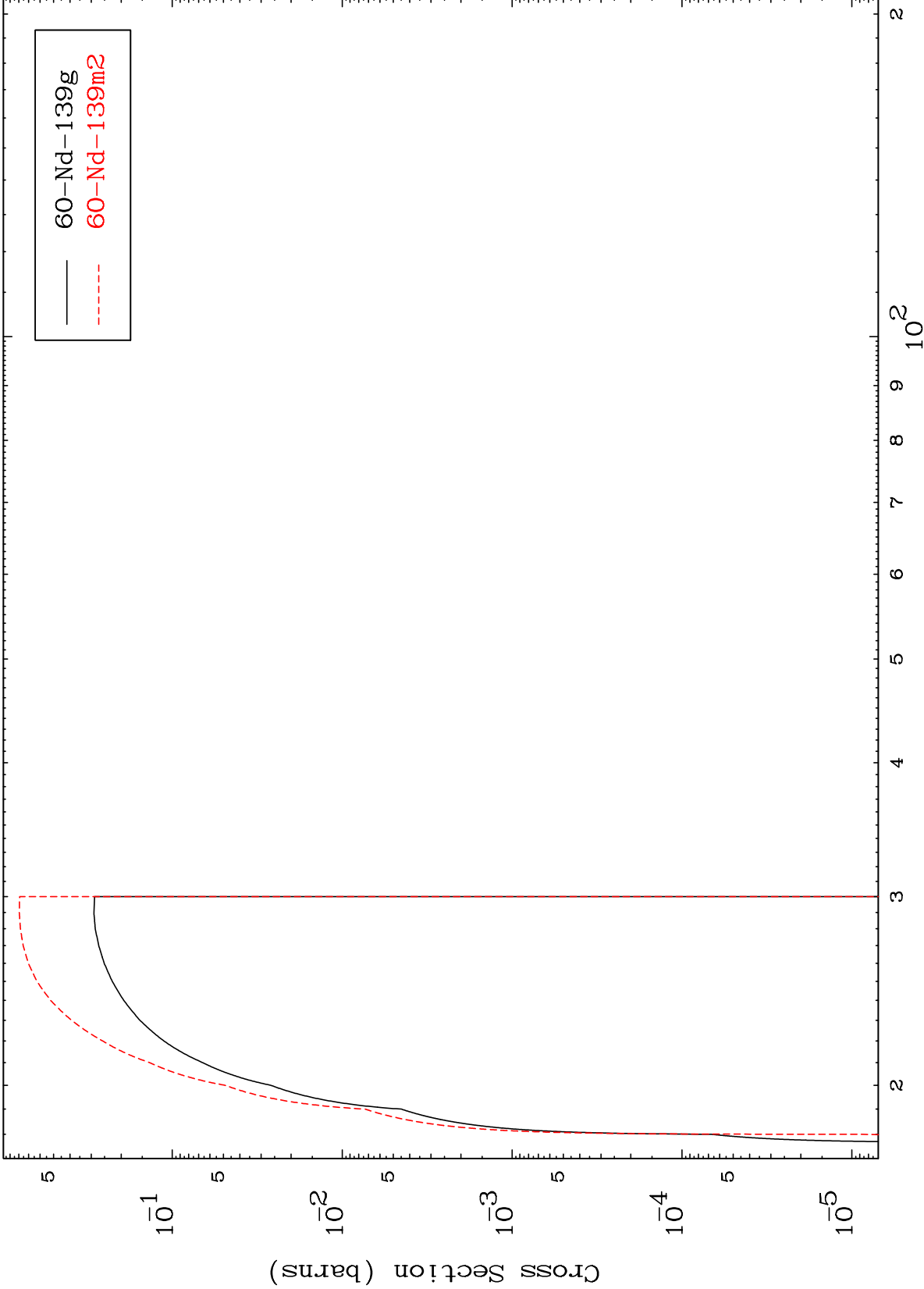


MAT 6023

(n,3n)

60-Nd-141m

Radionuclide Production Cross Section



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Incident Energy (MeV)

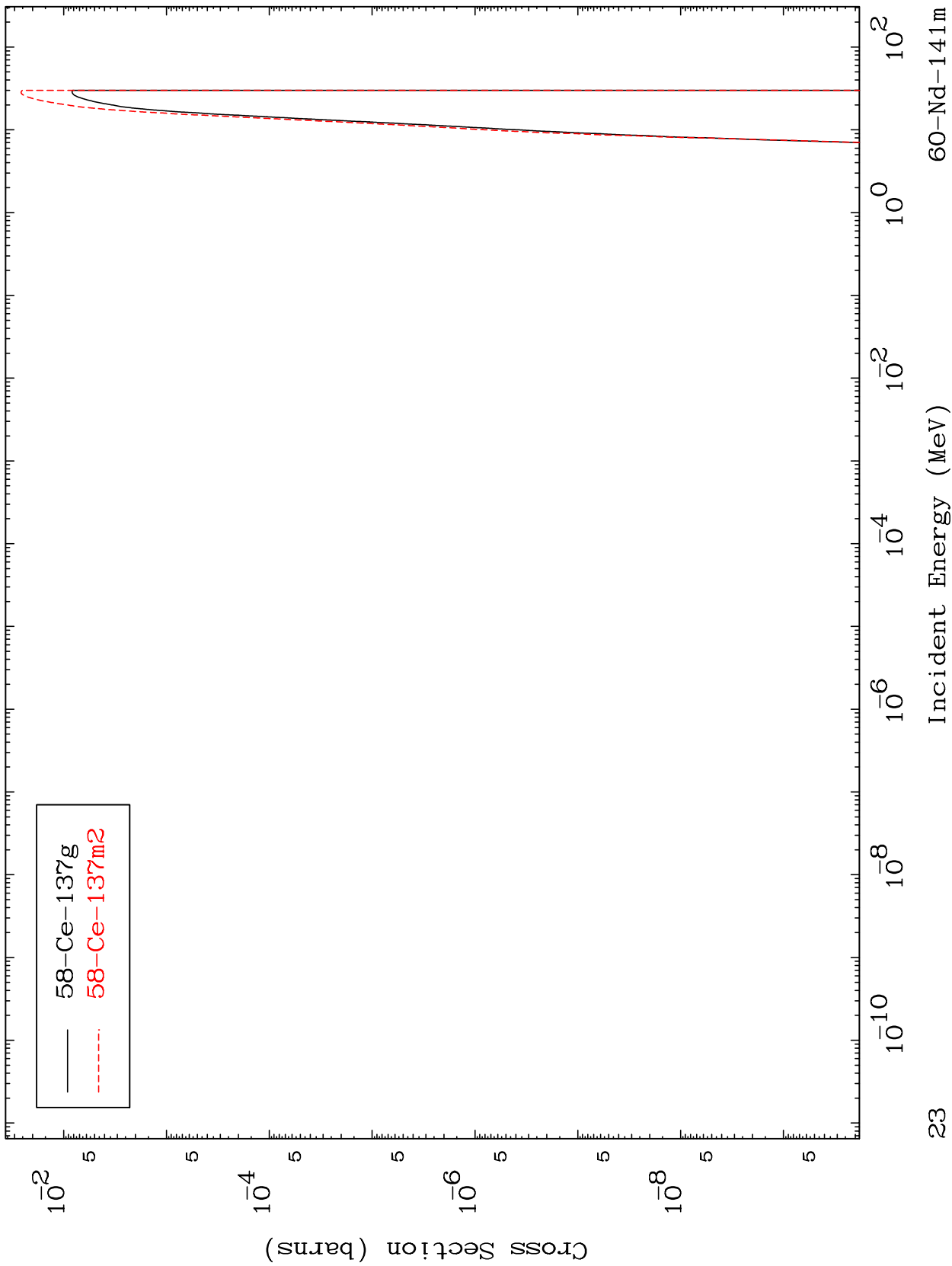
60-Nd-141m

MAT 6023

$(n, n') \alpha$

60-Nd-141m

Radionuclide Production Cross Section

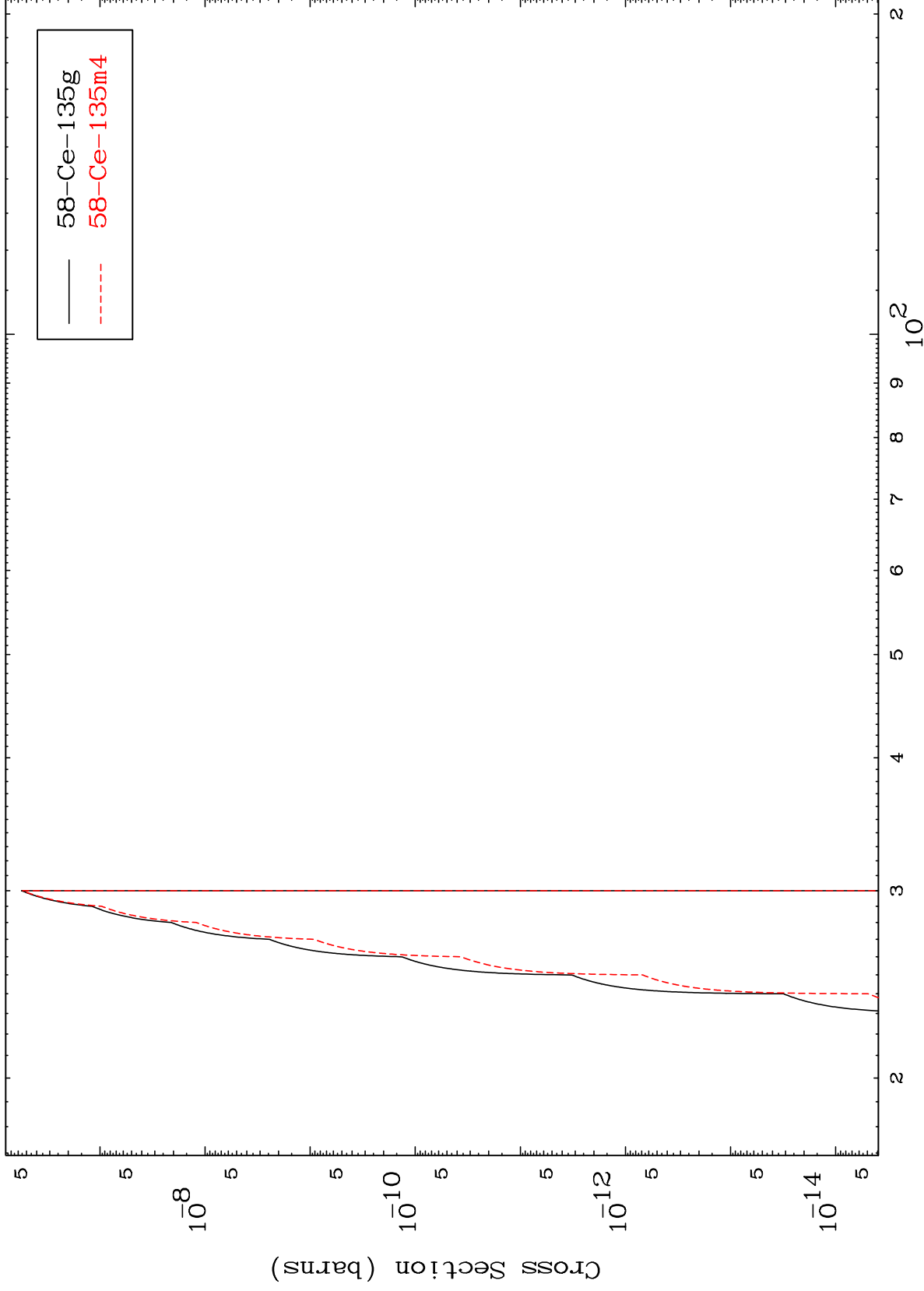


MAT 6023

(n,3n)  $\alpha$

60-Nd-141m

Radionuclide Production Cross Section



24

Incident Energy (MeV)

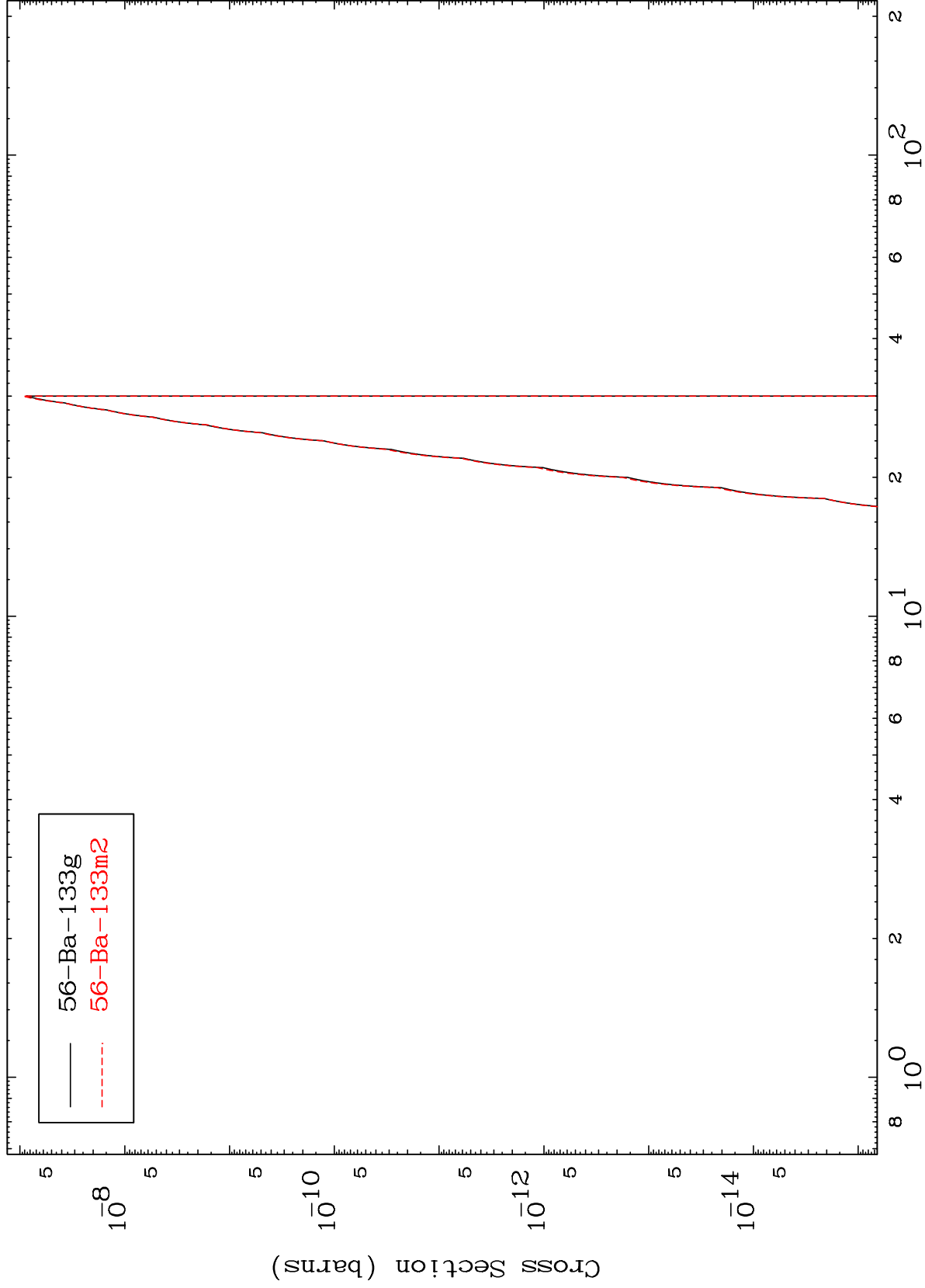
60-Nd-141m

MAT 6023

(n,n') 2α

60-Nd-141m

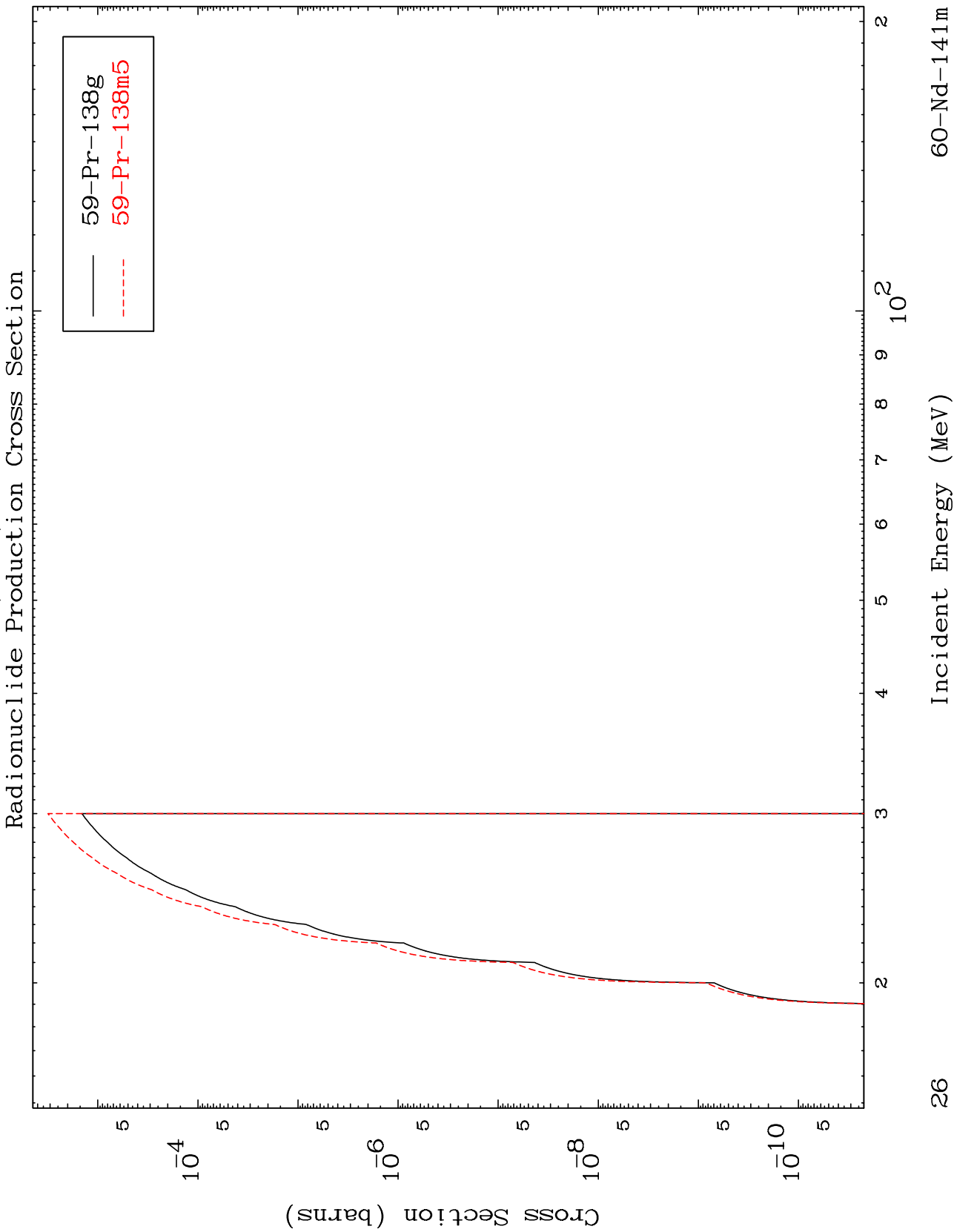
Radionuclide Production Cross Section



25

Incident Energy (MeV)

60-Nd-141m

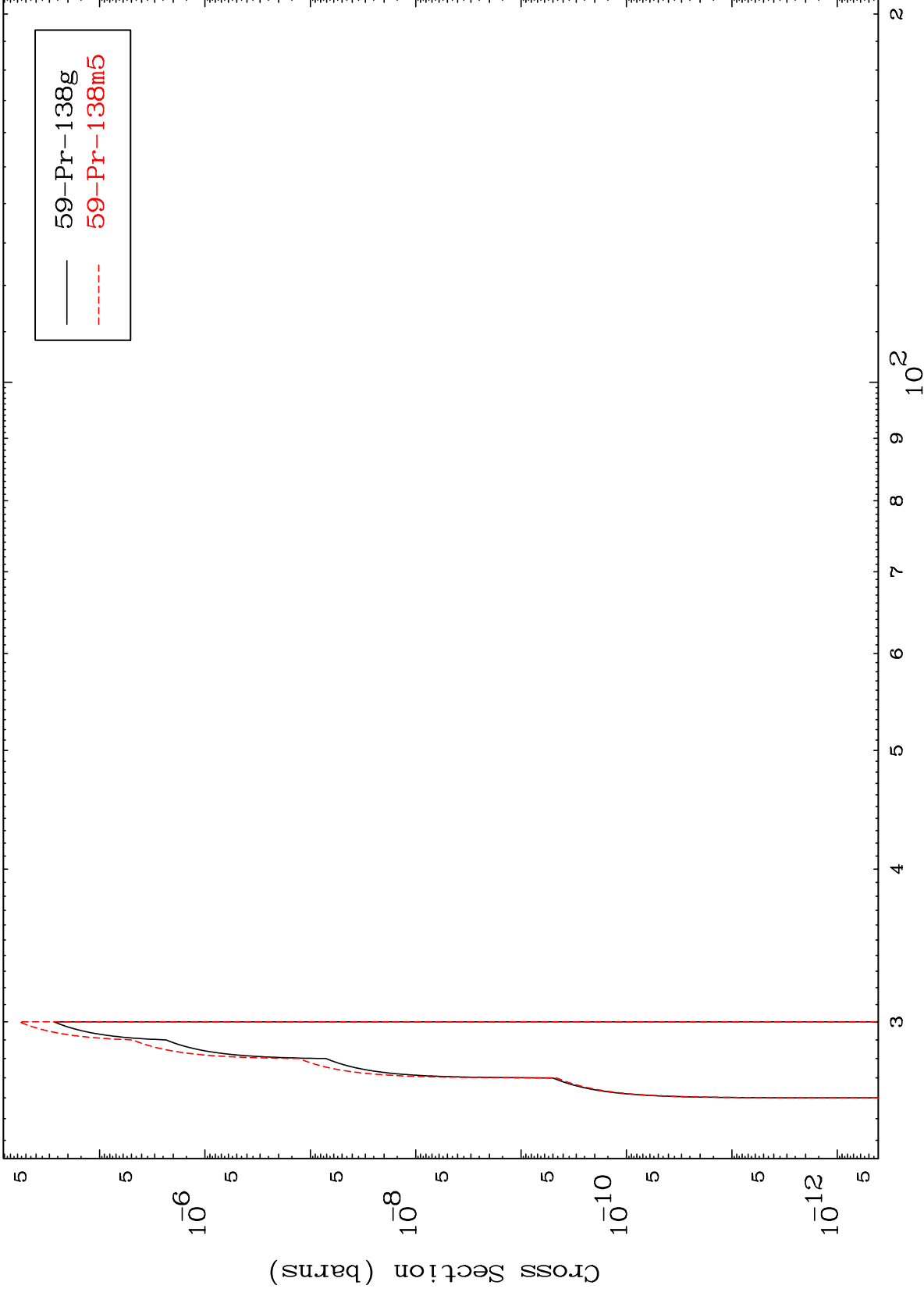


MAT 6023

(n,3n) p

60-Nd-141m

Radionuclide Production Cross Section



27

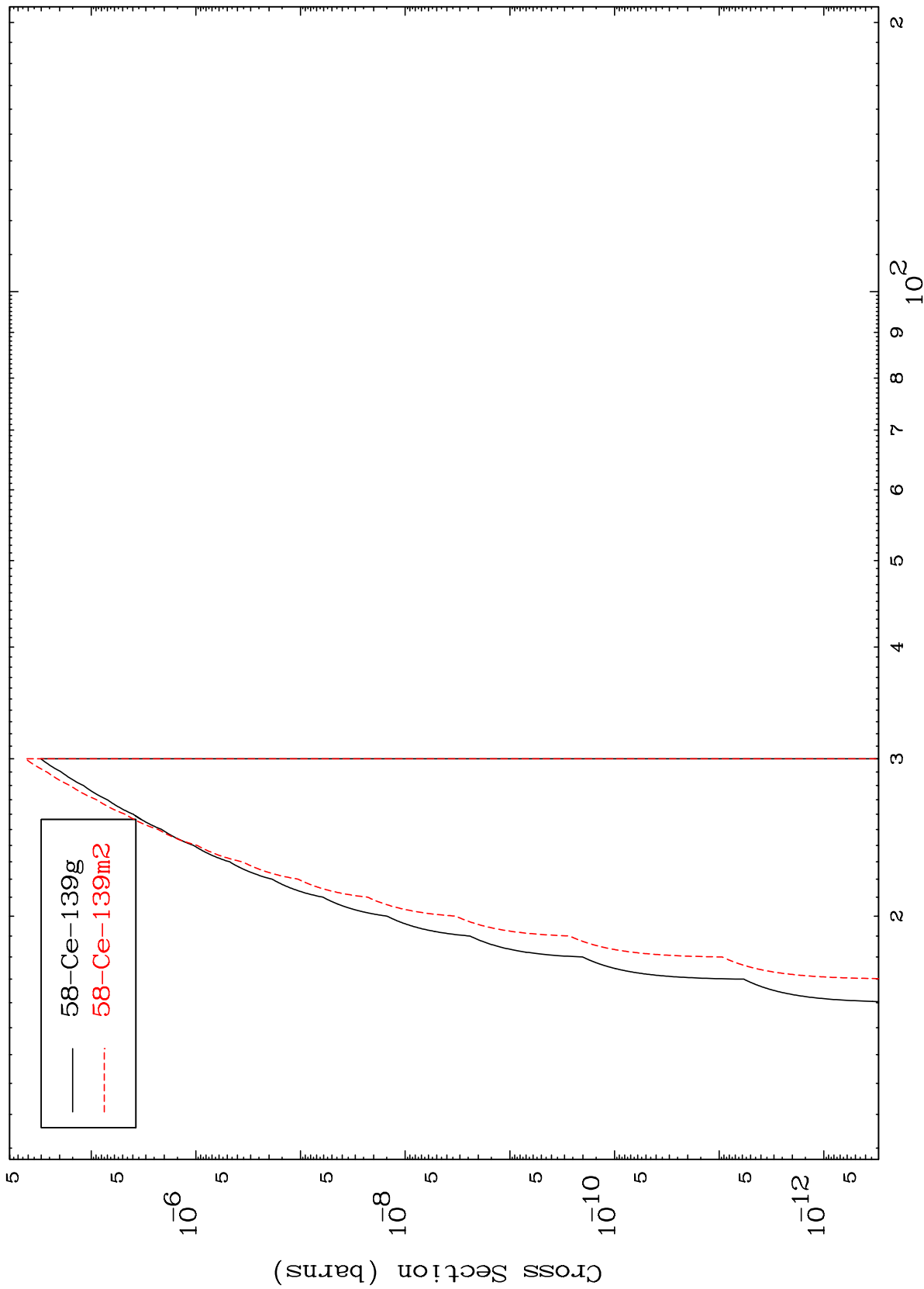
Incident Energy (MeV)

60-Nd-141m

MAT 6023

60-Nd-141m

(n,2n) p  
Radionuclide Production Cross Section



Incident Energy (MeV)

60-Nd-141m

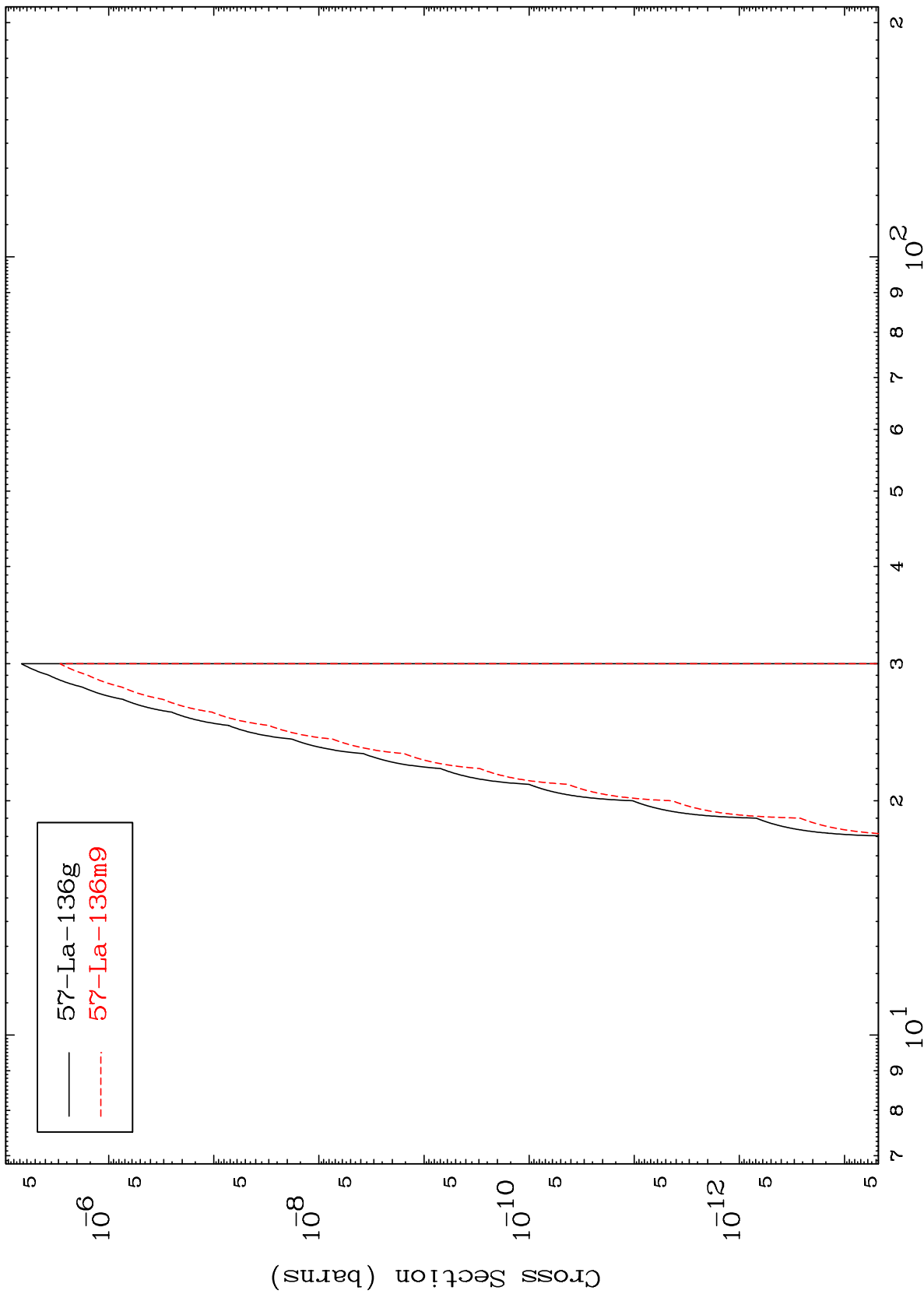
28

MAT 6023

(n,n') p  $\alpha$

60-Nd-141m

Radionuclide Production Cross Section



— 57-La-136g  
- - - 57-La-136m9

29

Incident Energy (MeV)

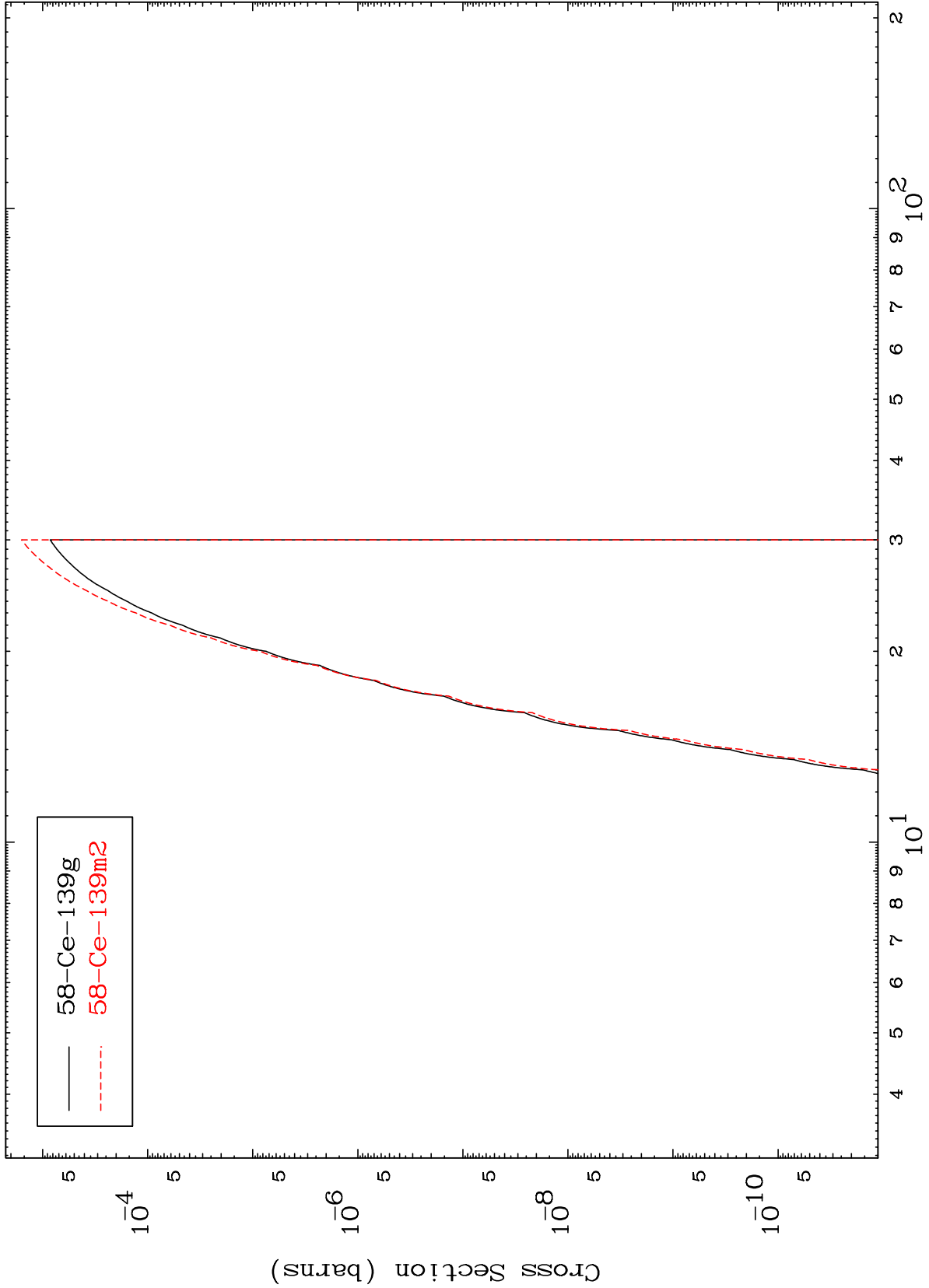
60-Nd-141m

MAT 6023

(n,He-3)

60-Nd-141m

Radionuclide Production Cross Section



30

Incident Energy (MeV)

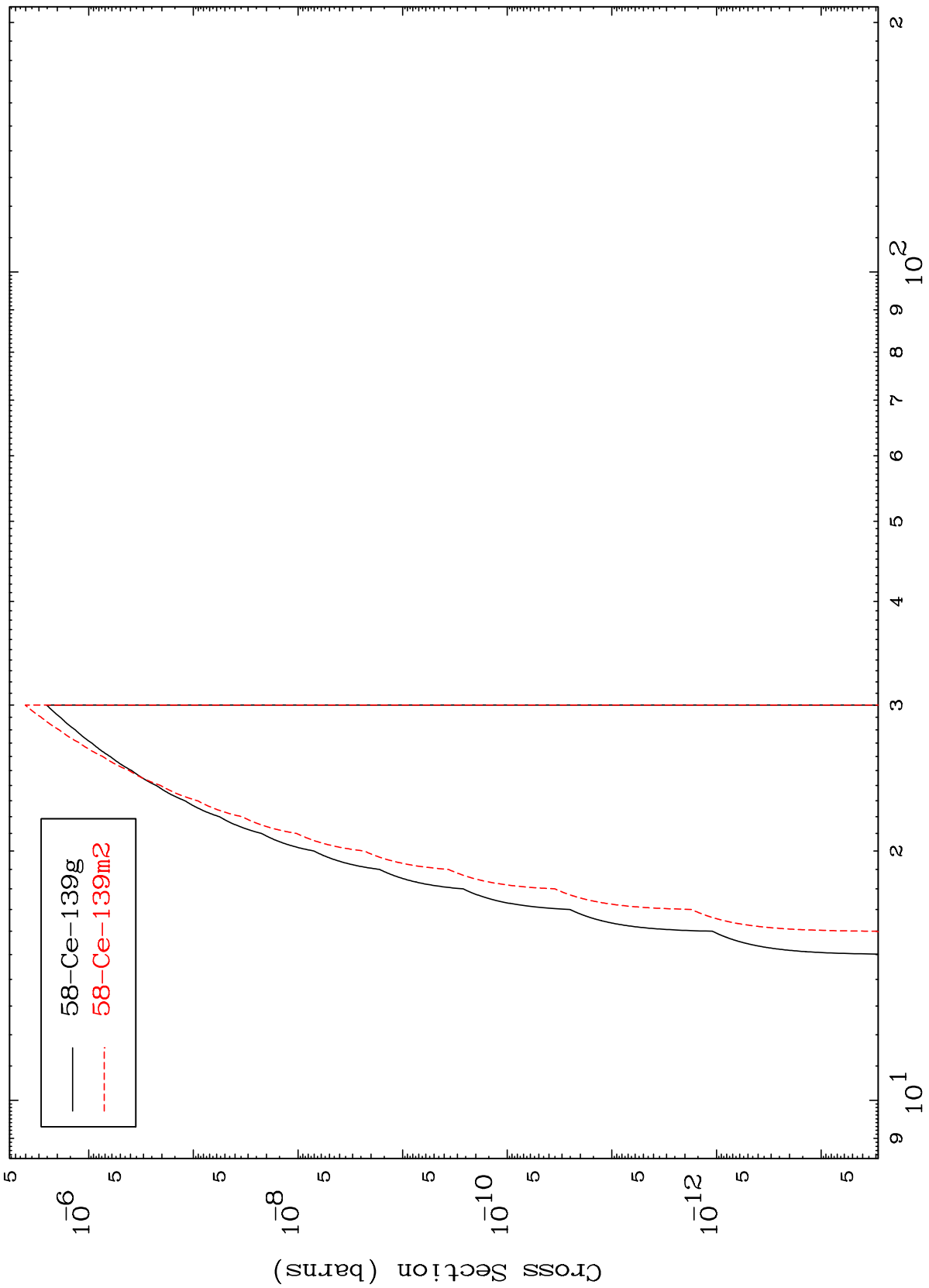
60-Nd-141m

MAT 6023

(n,p) d

60-Nd-141m

Radionuclide Production Cross Section



58-Ce-139g  
58-Ce-139m2

Incident Energy (MeV)

60-Nd-141m

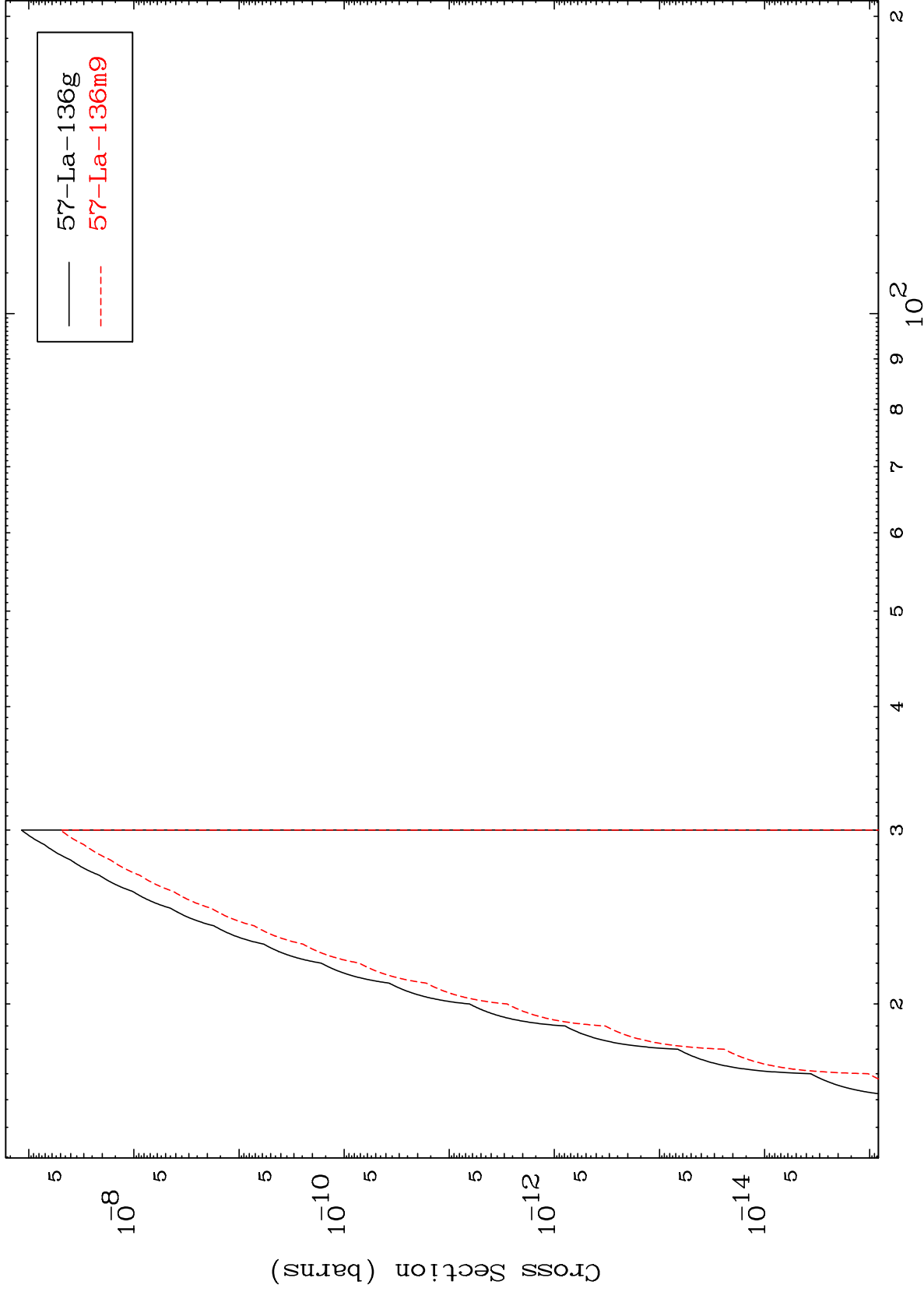
31

MAT 6023

(n,d)  $\alpha$

60-Nd-141m

Radionuclide Production Cross Section



32

Incident Energy (MeV)

60-Nd-141m