

Program EVALPLOT  
(Version 2021-1)

by

Dermott E. Cullen  
(Present Contact Information)

Dermott E. Cullen  
1466 Hudson Way  
Livermore, CA 94550  
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

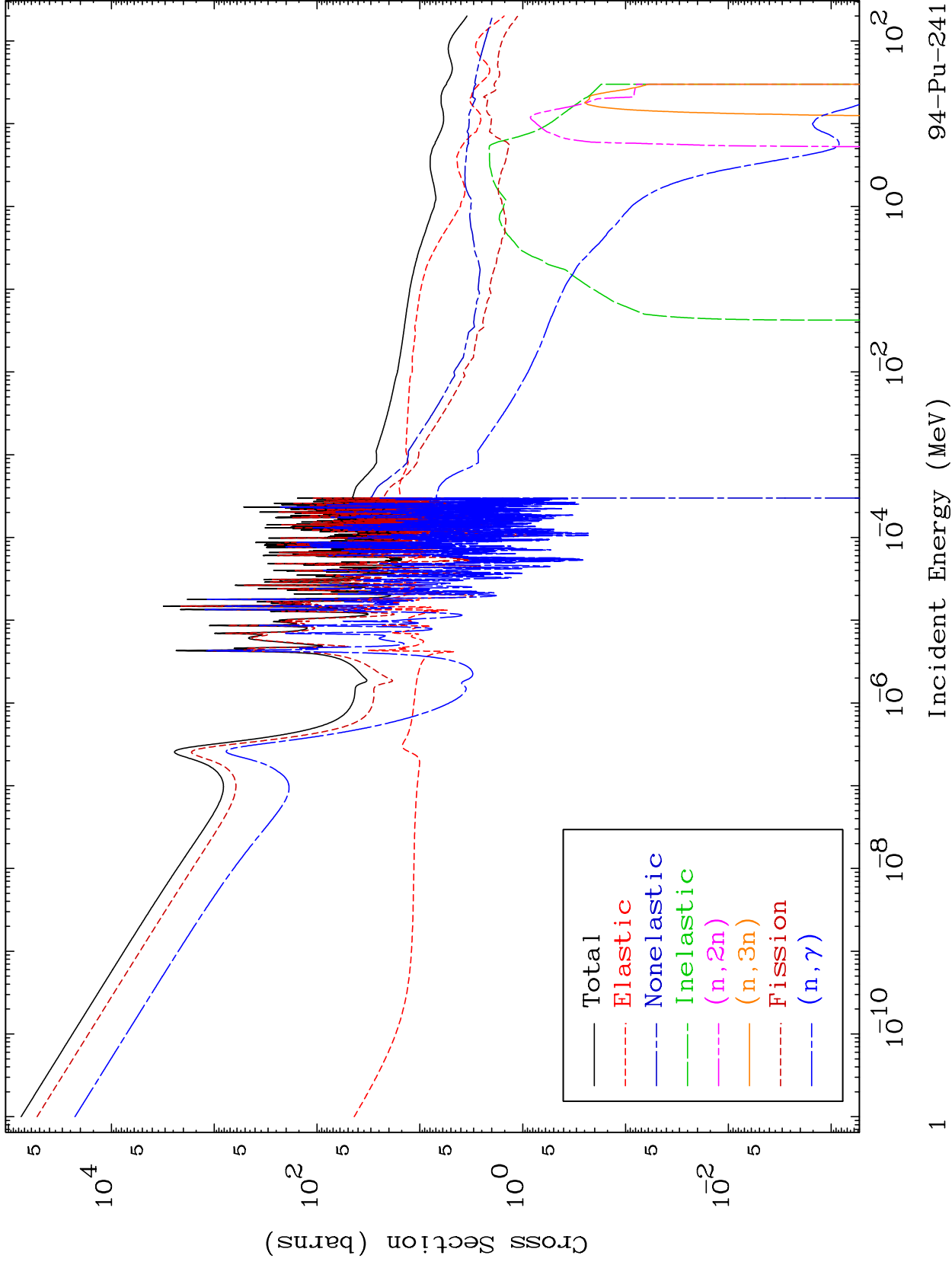
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 9443

Neutron Major  
293 Kelvin Cross Sections

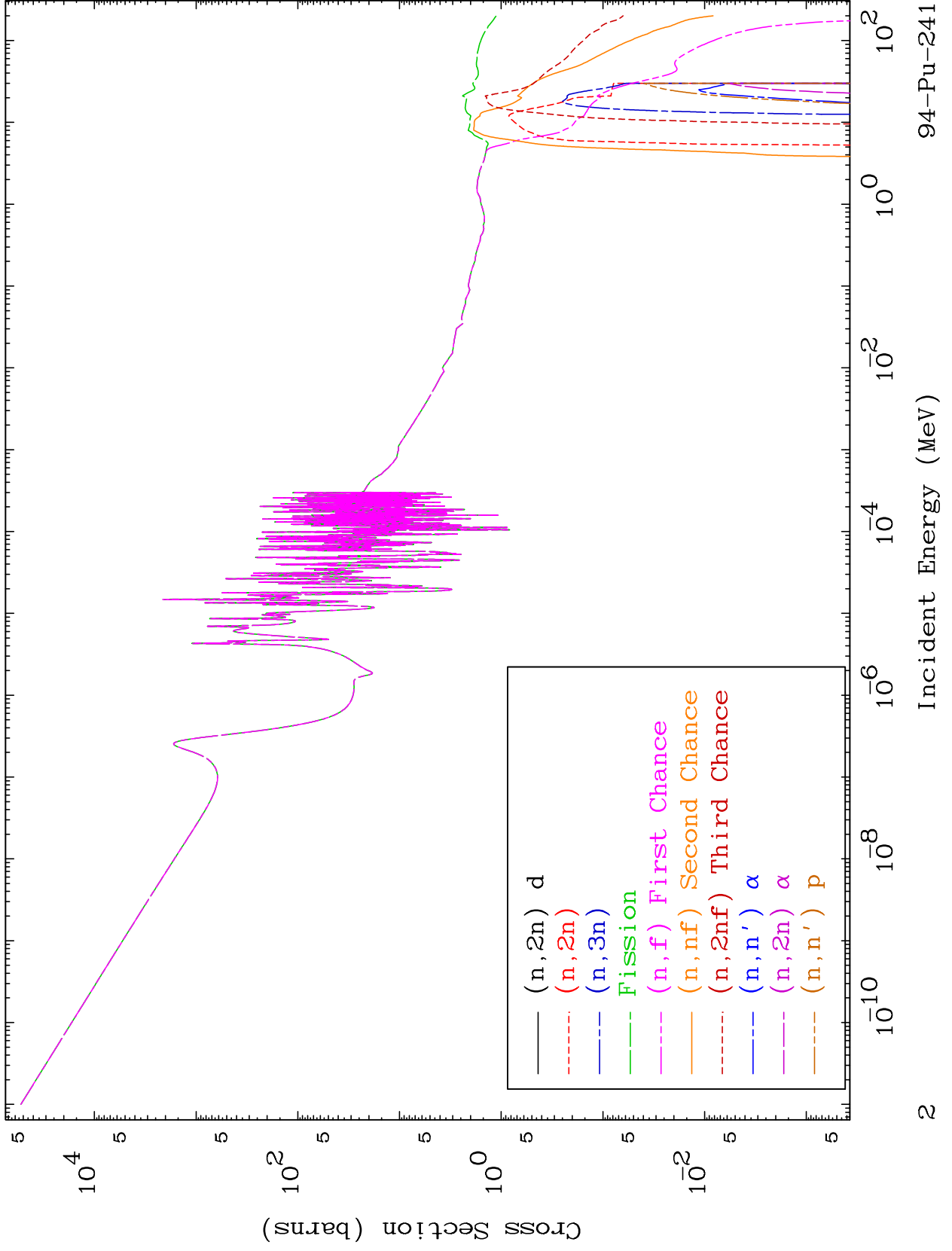
94-Pu-241



MAT 9443

Neutron Absorption  
293 Kelvin Cross Sections

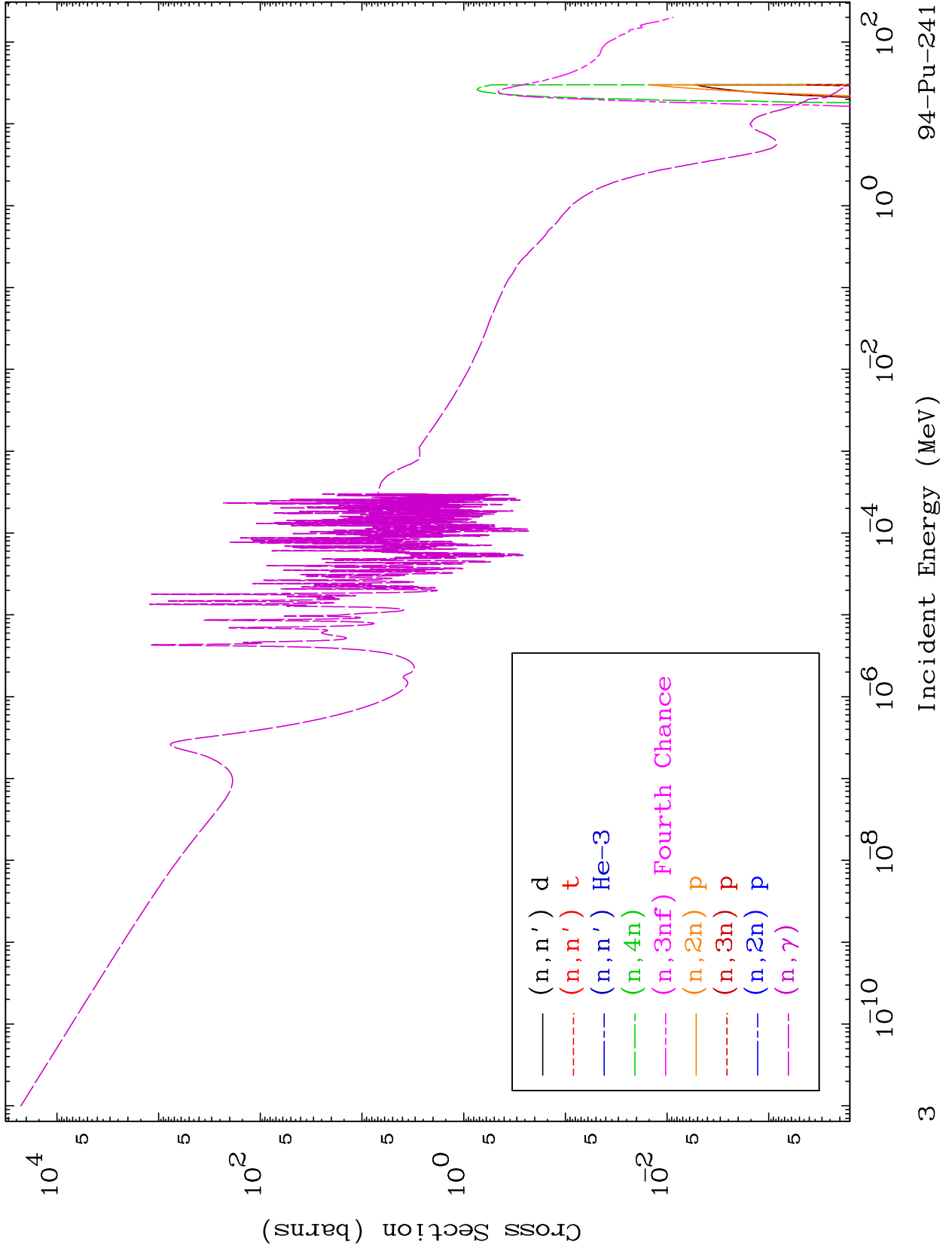
94-Pu-241



MAT 9443

Neutron Absorption  
293 Kelvin Cross Sections

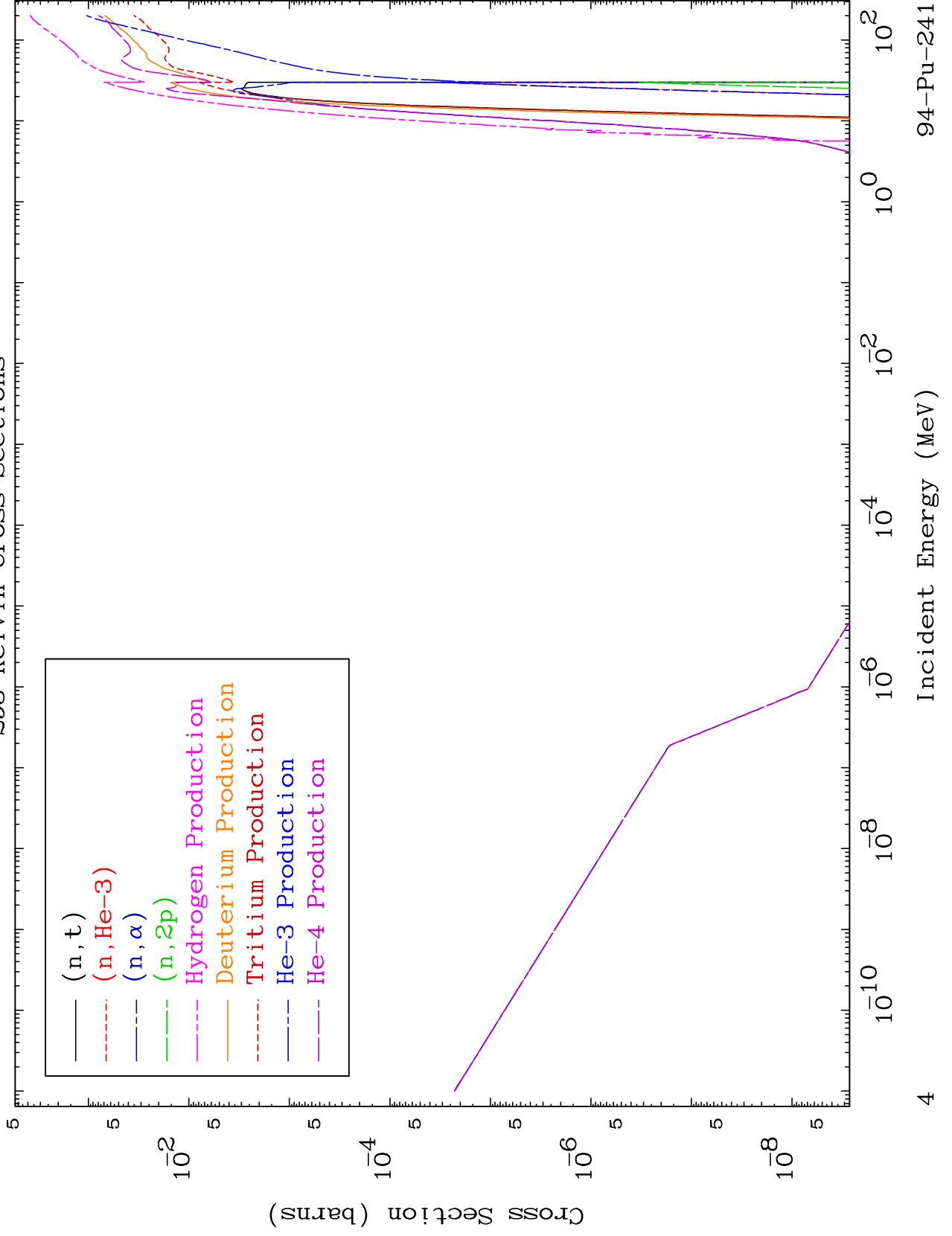
94-Pu-241



MAT 9443

Neutron Absorption  
293 Kelvin Cross Sections

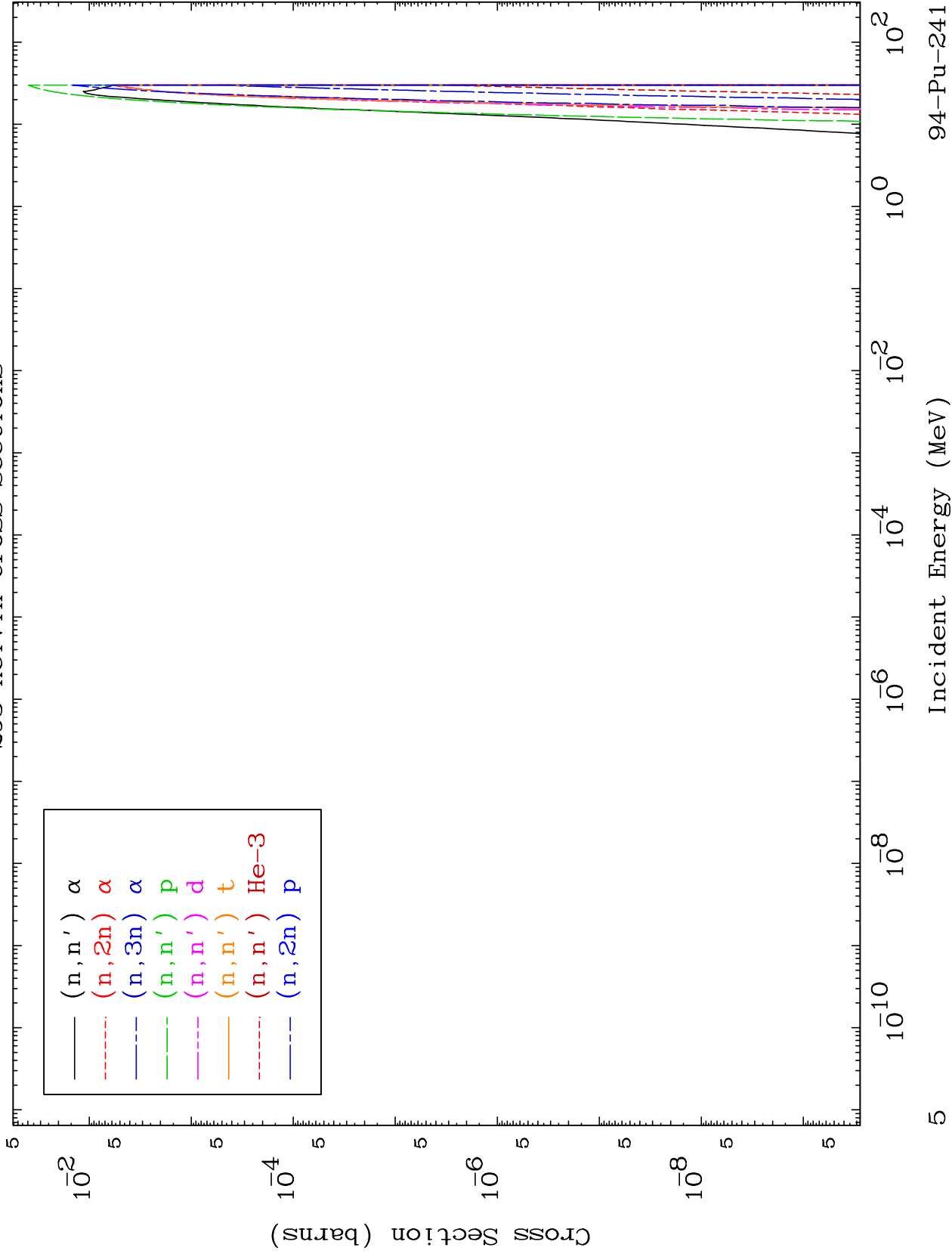
94-Pu-241



MAT 9443

Charged Particle  
293 Kelvin Cross Sections

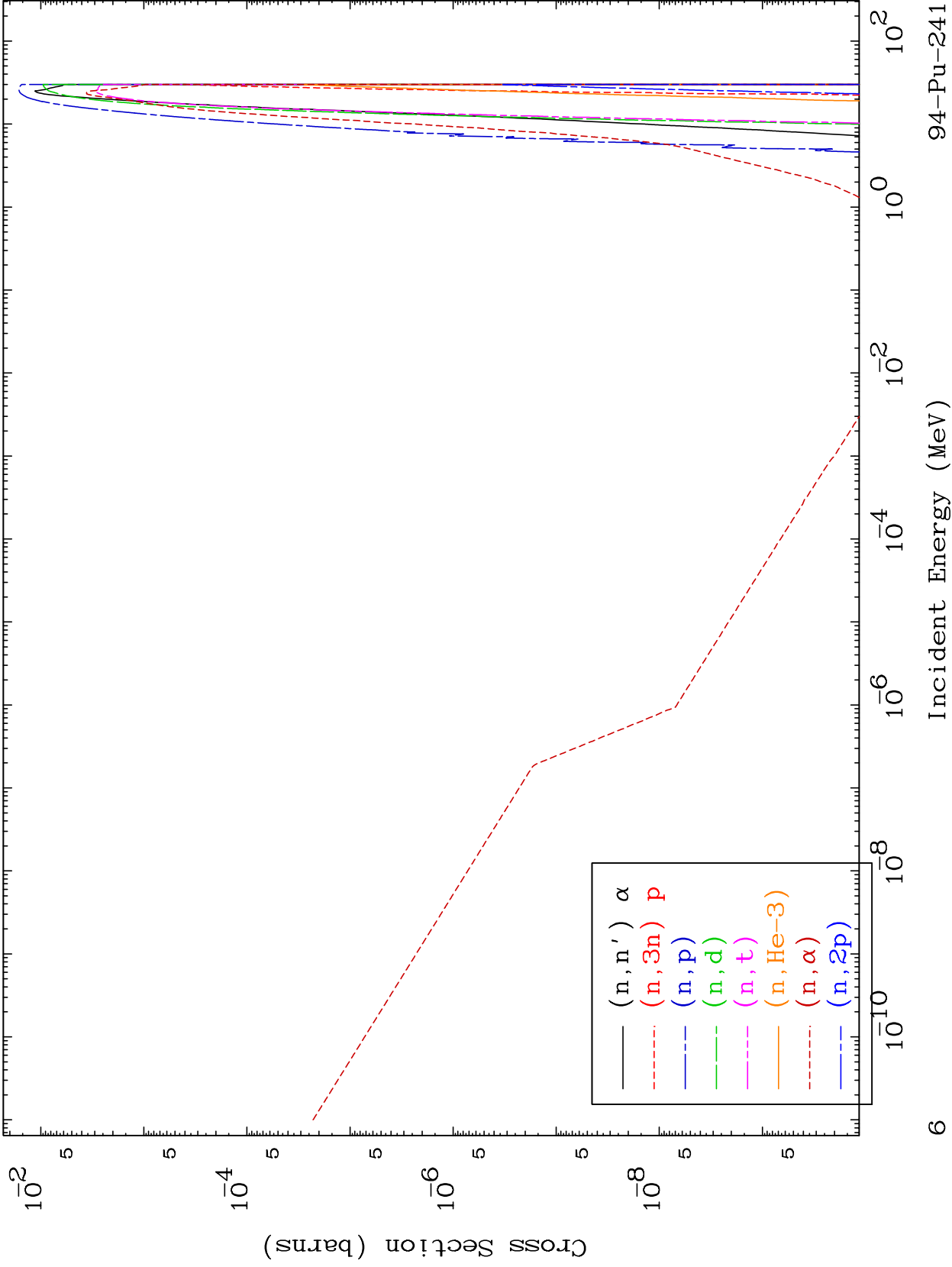
94-Pu-241



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Charged Particle  
293 Kelvin Cross Sections

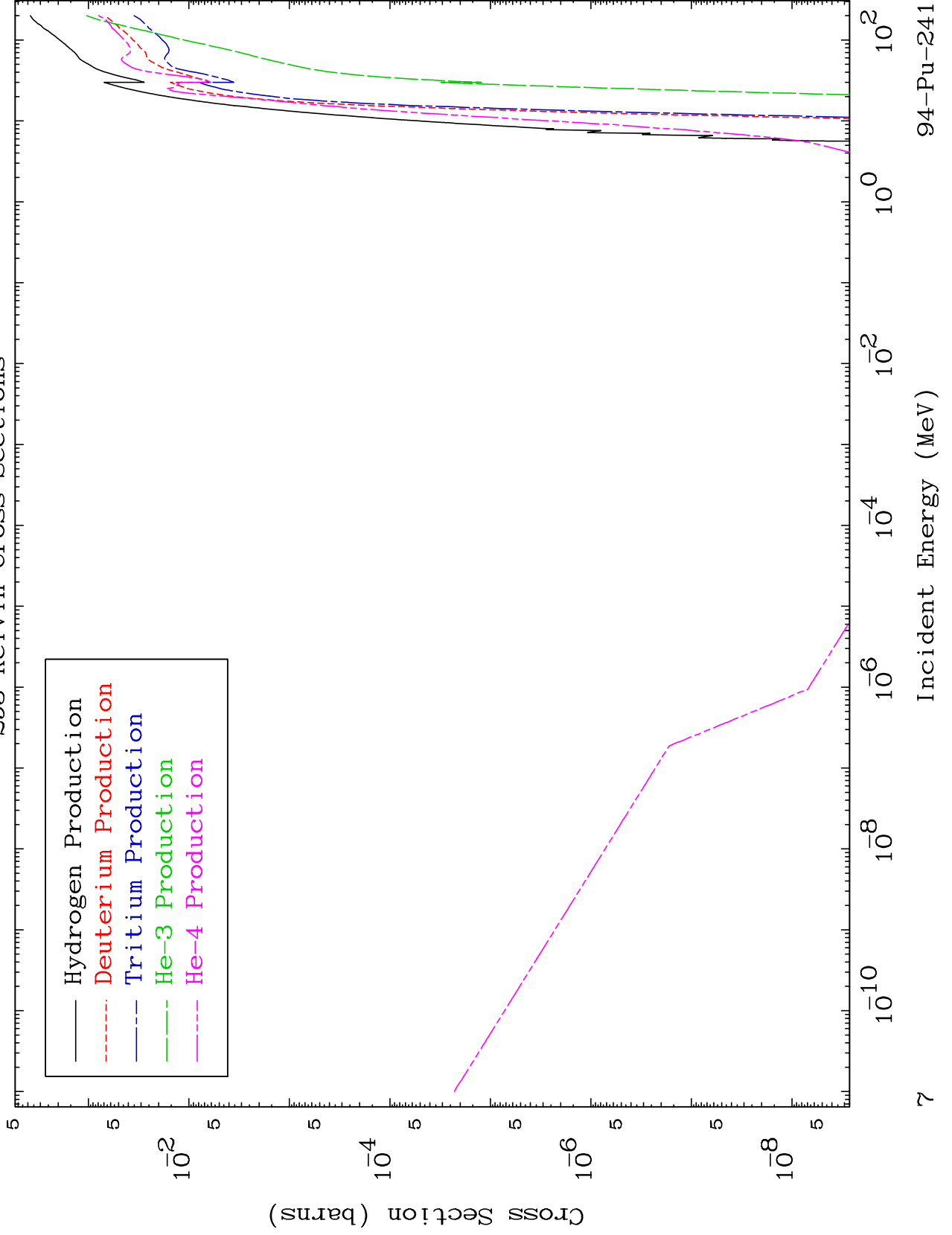
94-Pu-241



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Particle Production  
293 Kelvin Cross Sections

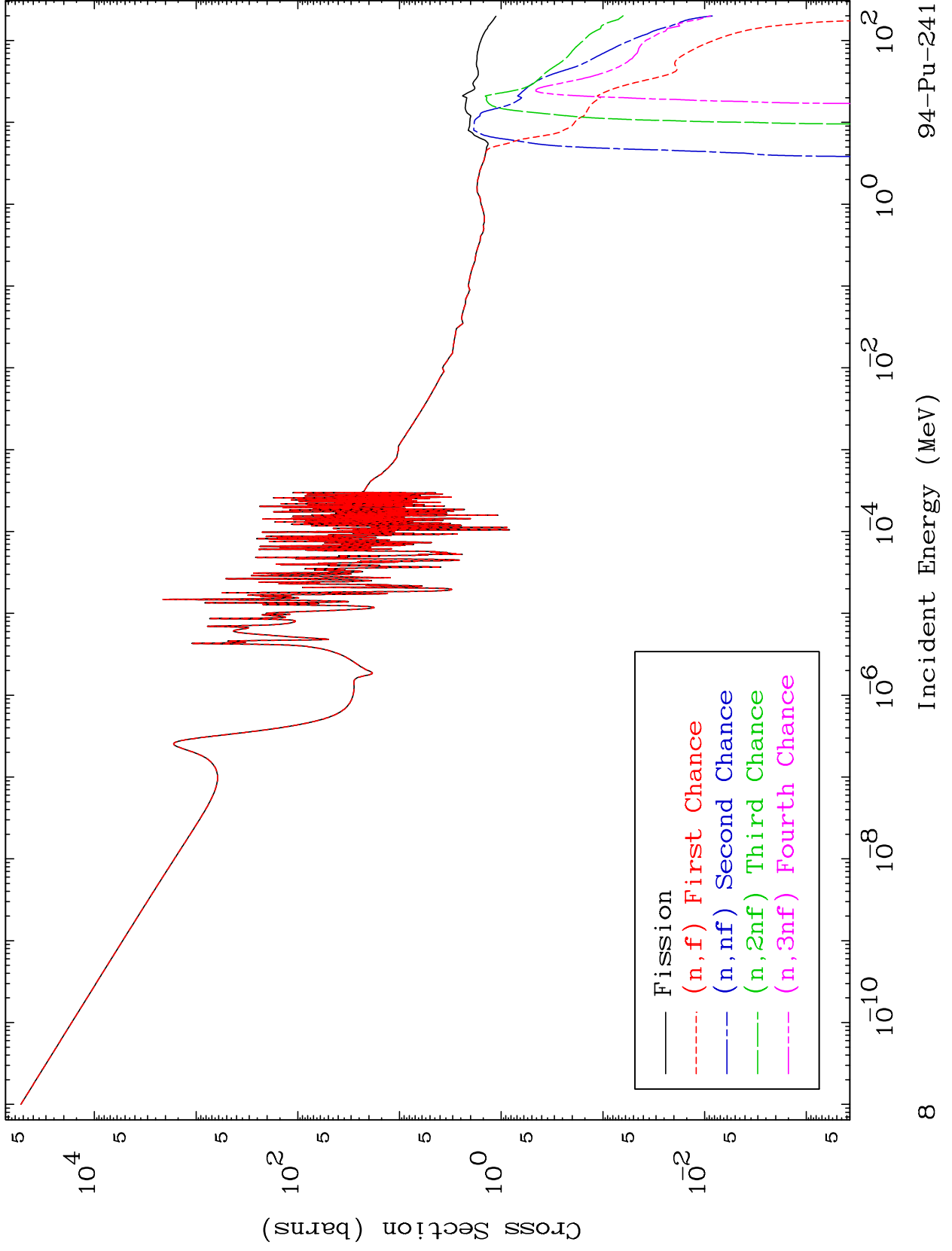
94-Pu-241



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Fission  
293 Kelvin Cross Sections

94-Pu-241

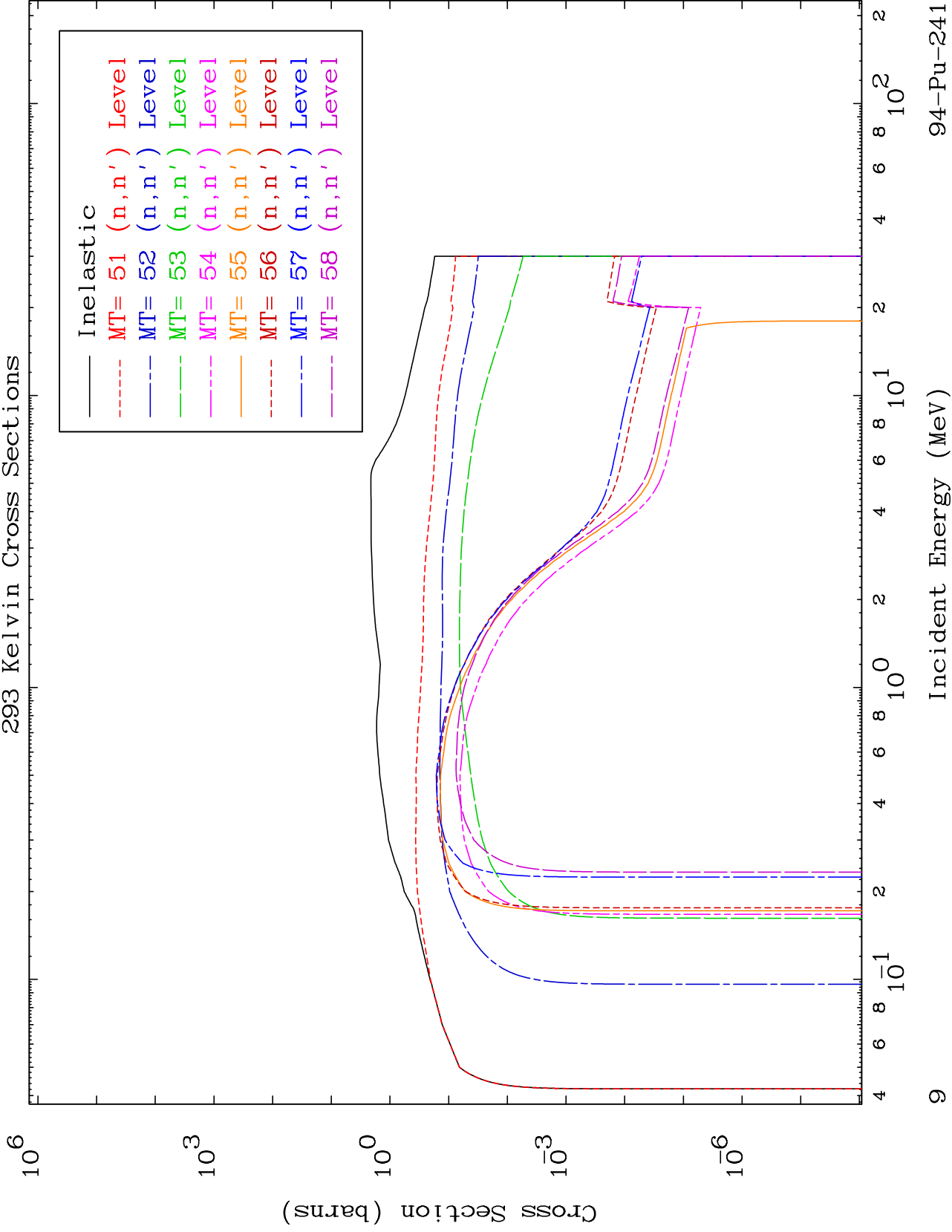


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(n,n') Levels

94-Pu-241

293 Kelvin Cross Sections

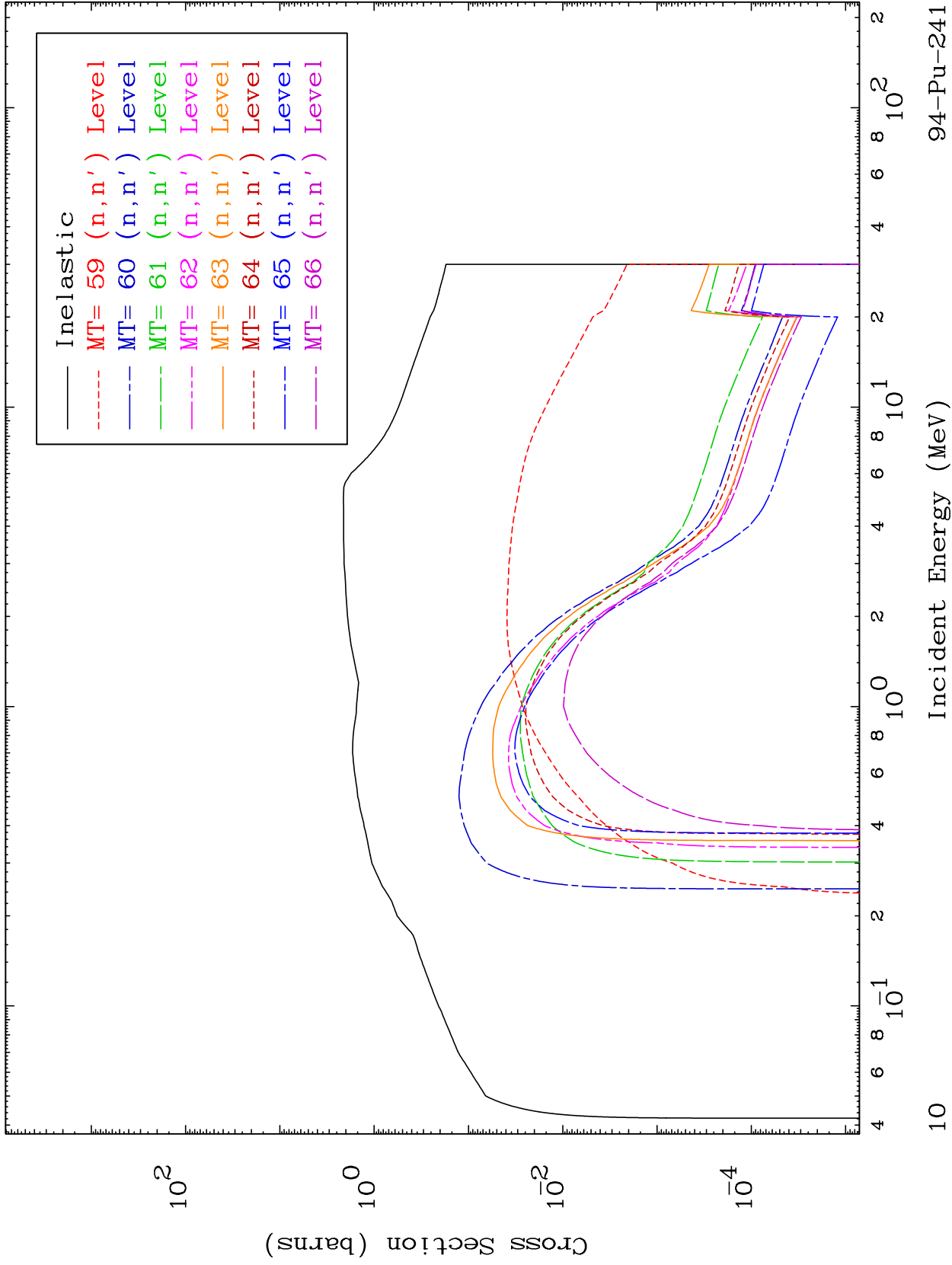


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(n,n') Levels

94-Pu-241

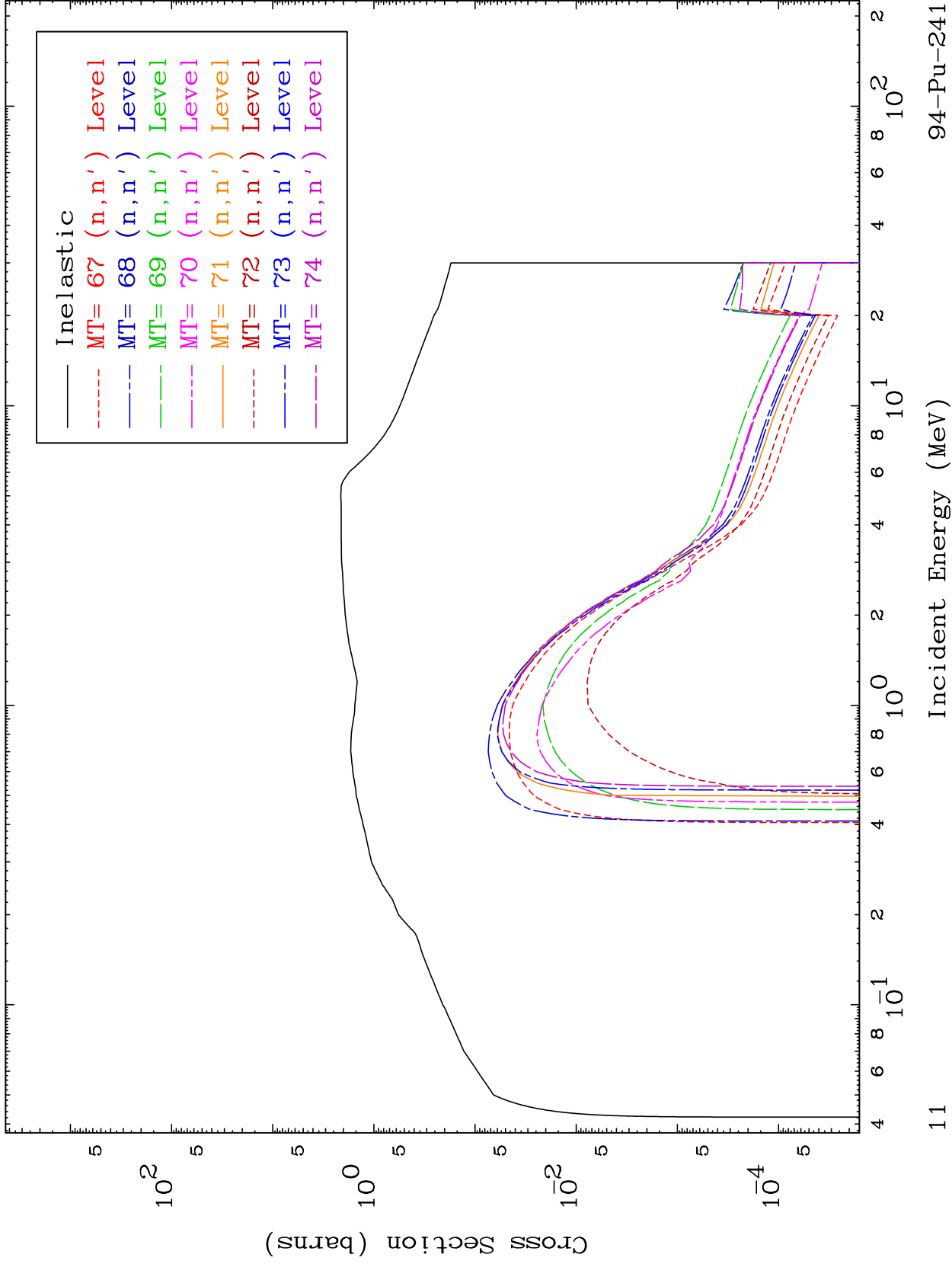
293 Kelvin Cross Sections



10

Incident Energy (MeV)

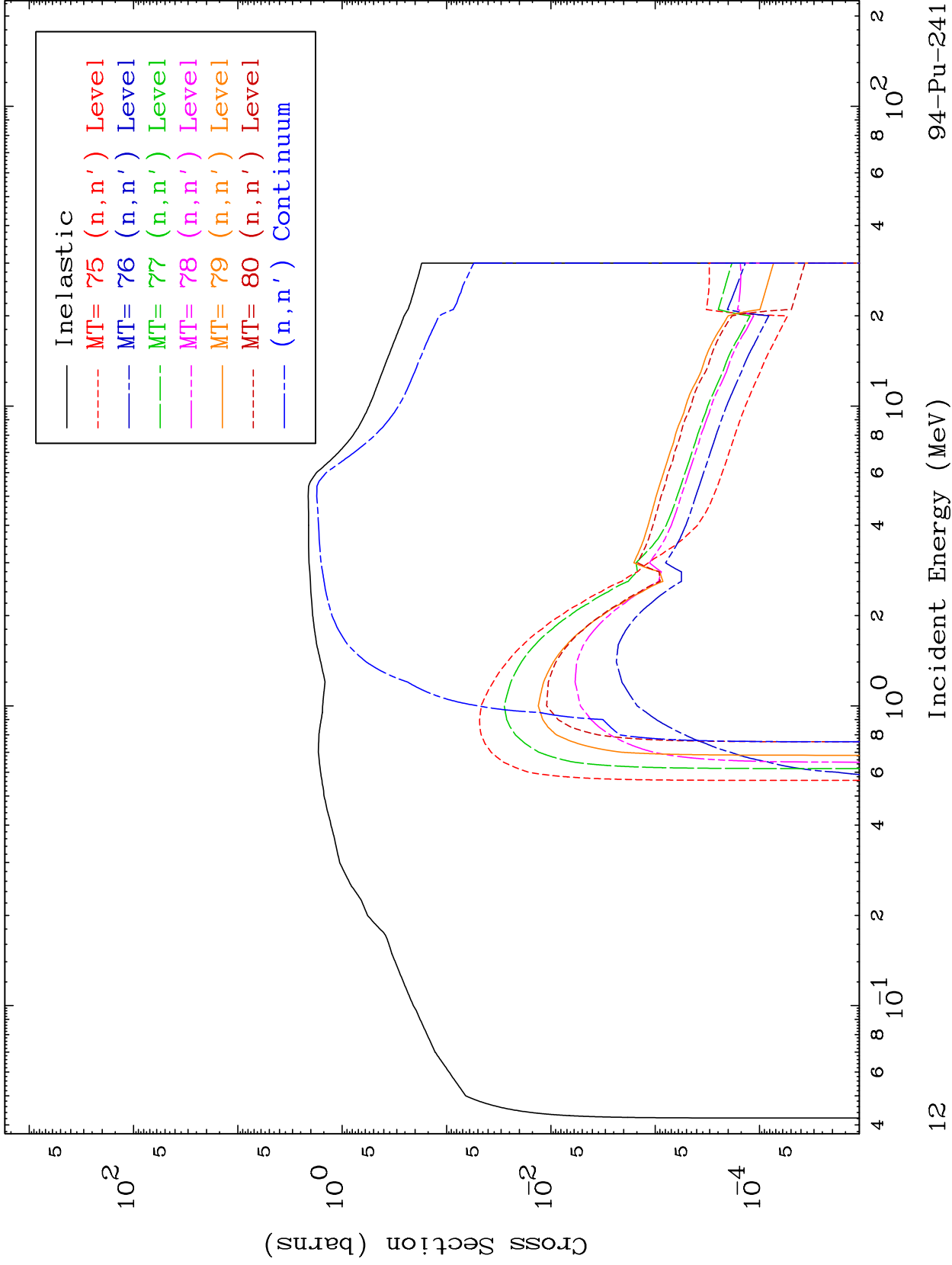
94-Pu-241



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(n,n') Levels  
293 Kelvin Cross Sections

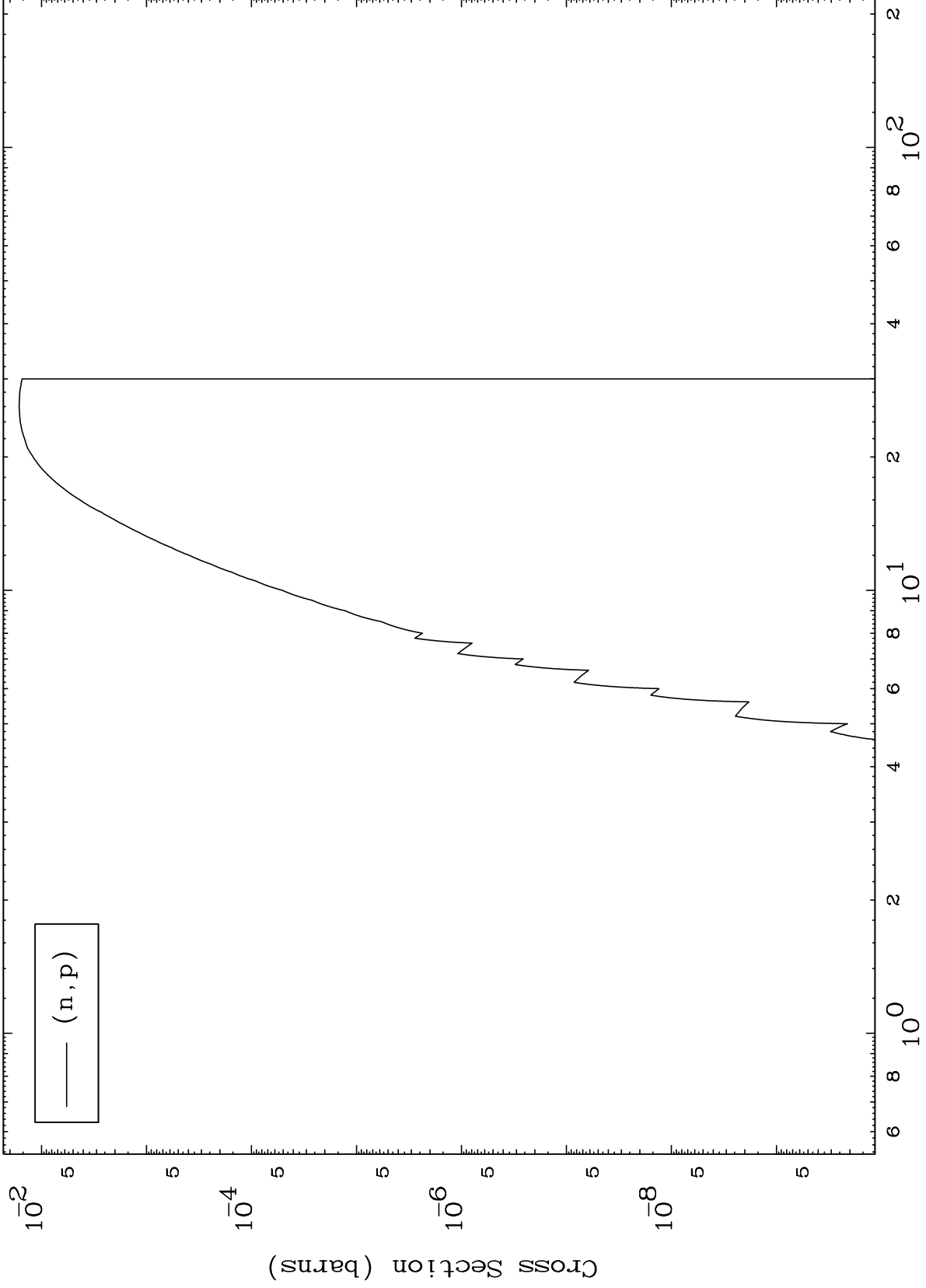
94-Pu-241



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(n,p) Levels  
293 Kelvin Cross Sections

94-Pu-241



13

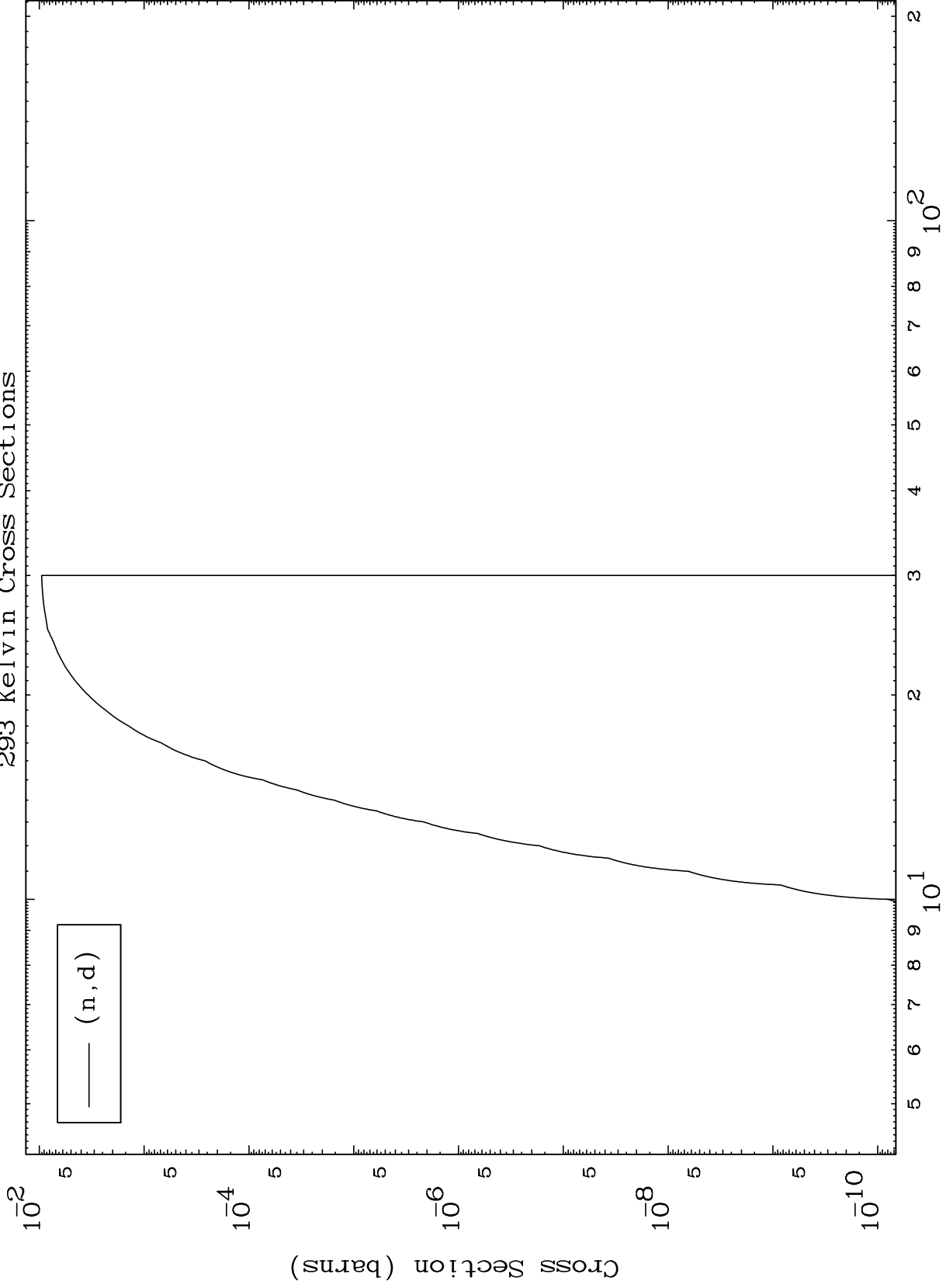
Incident Energy (MeV)

94-Pu-241

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(n,d) Levels  
293 Kelvin Cross Sections

94-Pu-241



14

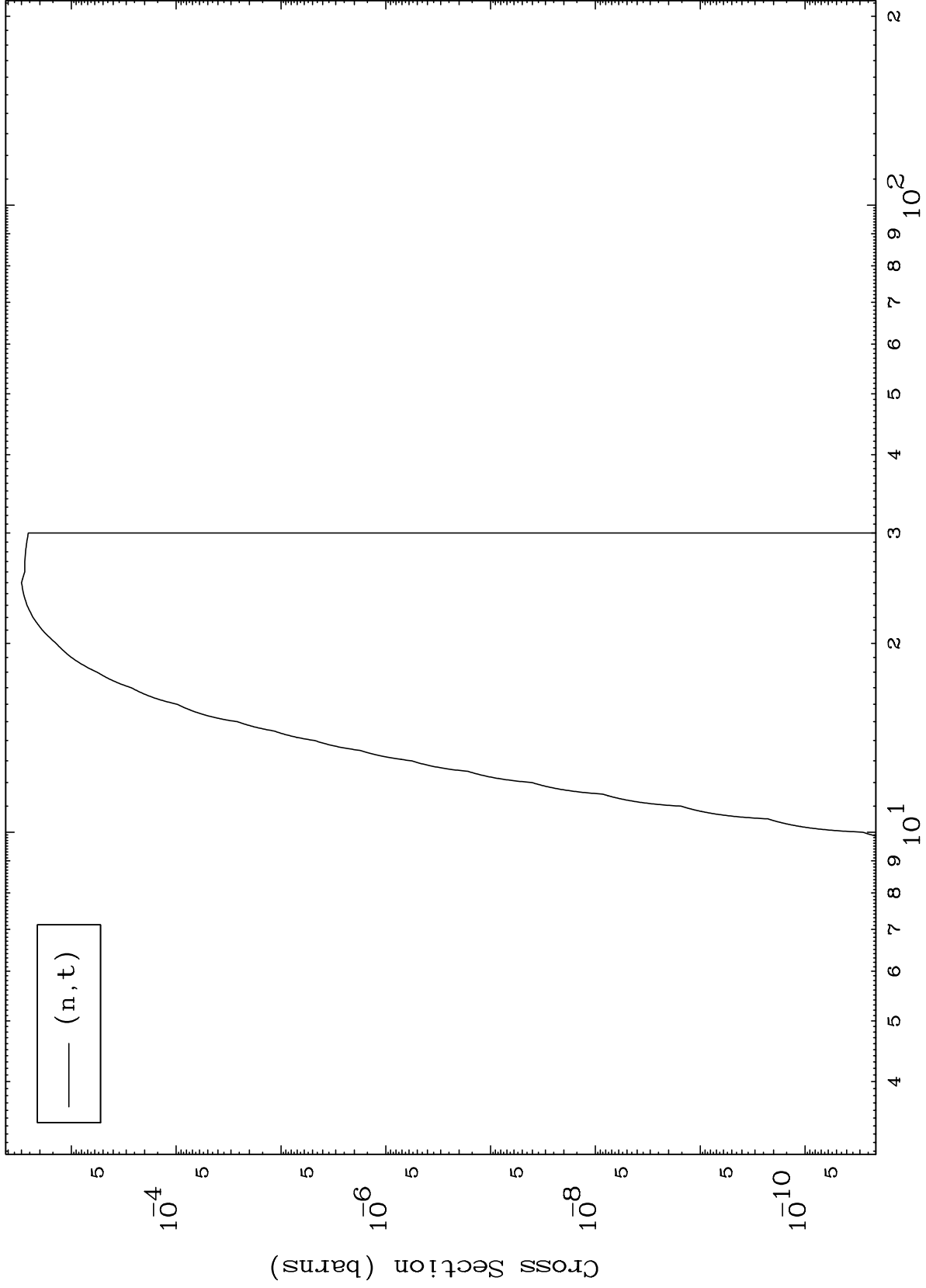
Incident Energy (MeV)

94-Pu-241

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(n,t) Levels  
293 Kelvin Cross Sections

94-Pu-241



15

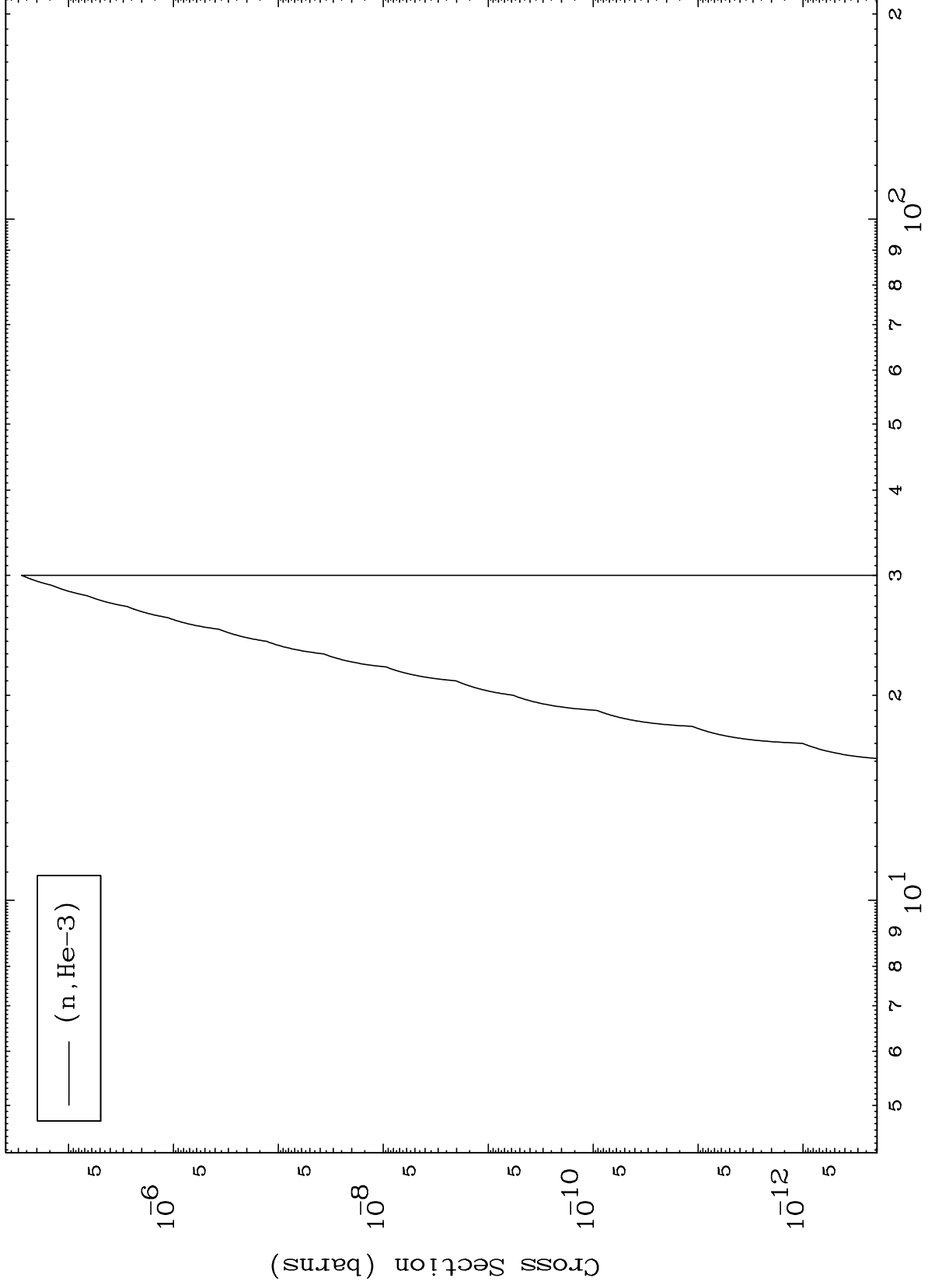
Incident Energy (MeV)

94-Pu-241

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(n,He3) Levels  
293 Kelvin Cross Sections

94-Pu-241



16

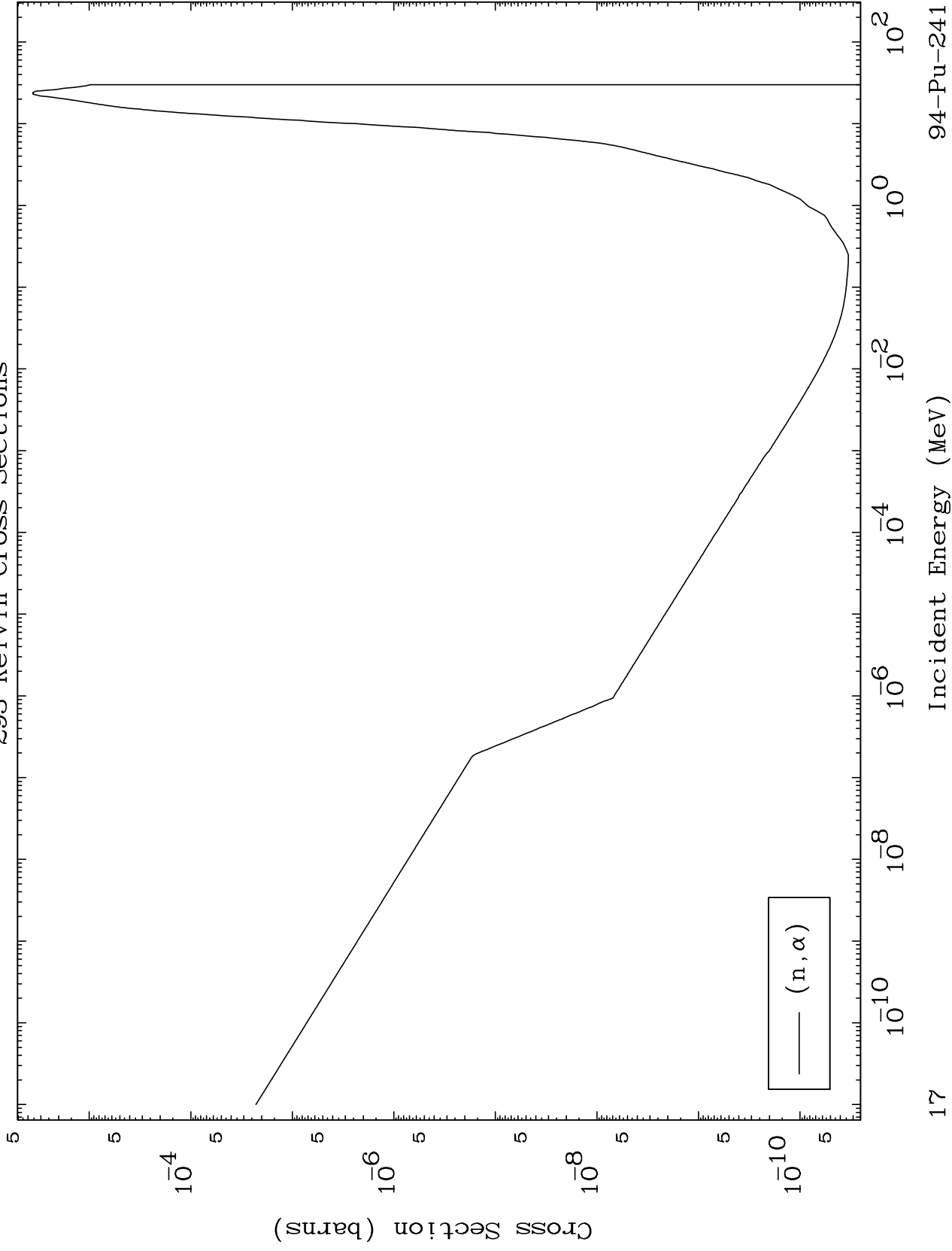
Incident Energy (MeV)

94-Pu-241

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(n,  $\alpha$ ) Levels  
293 Kelvin Cross Sections

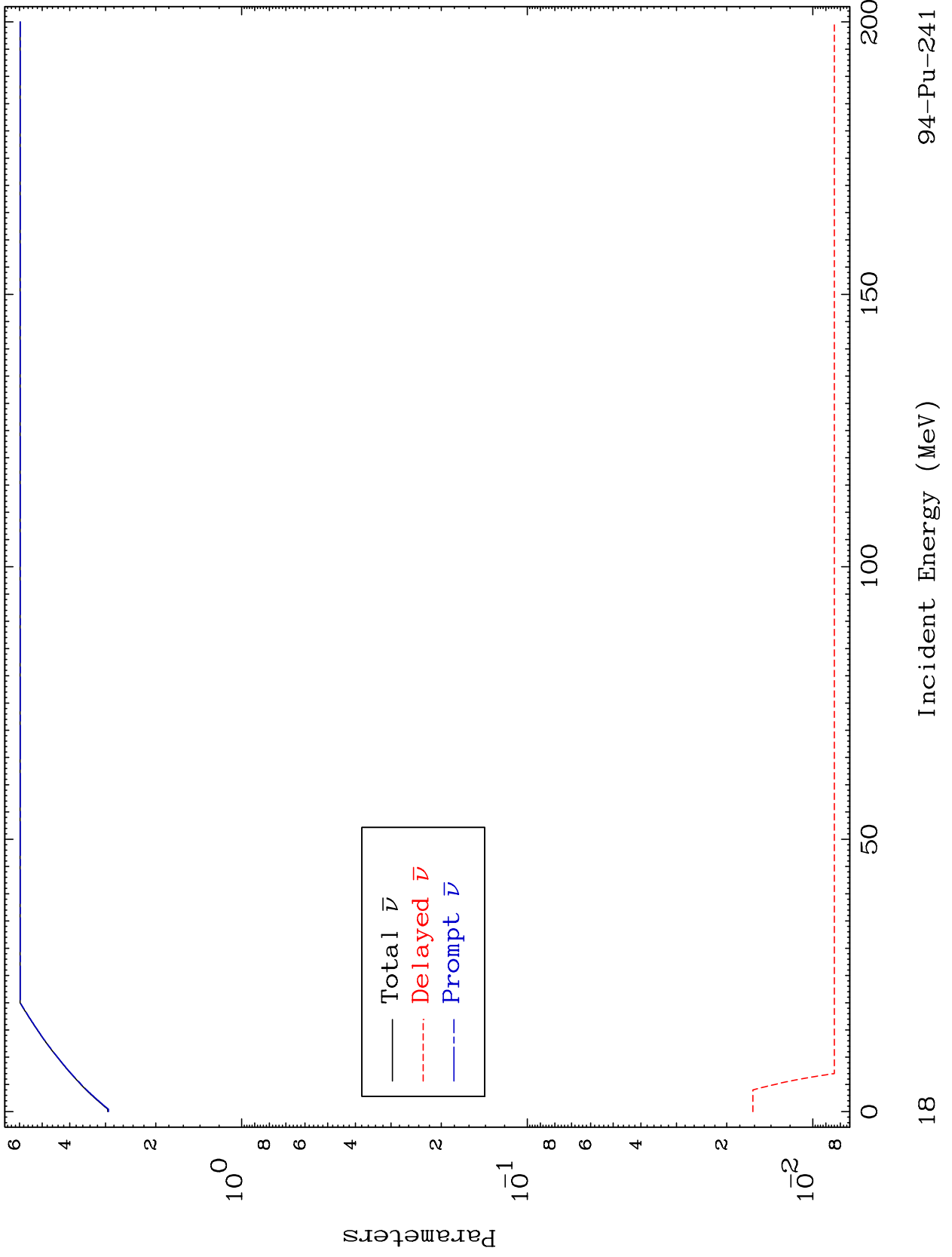
94-Pu-241



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Energy Release  
Parameters

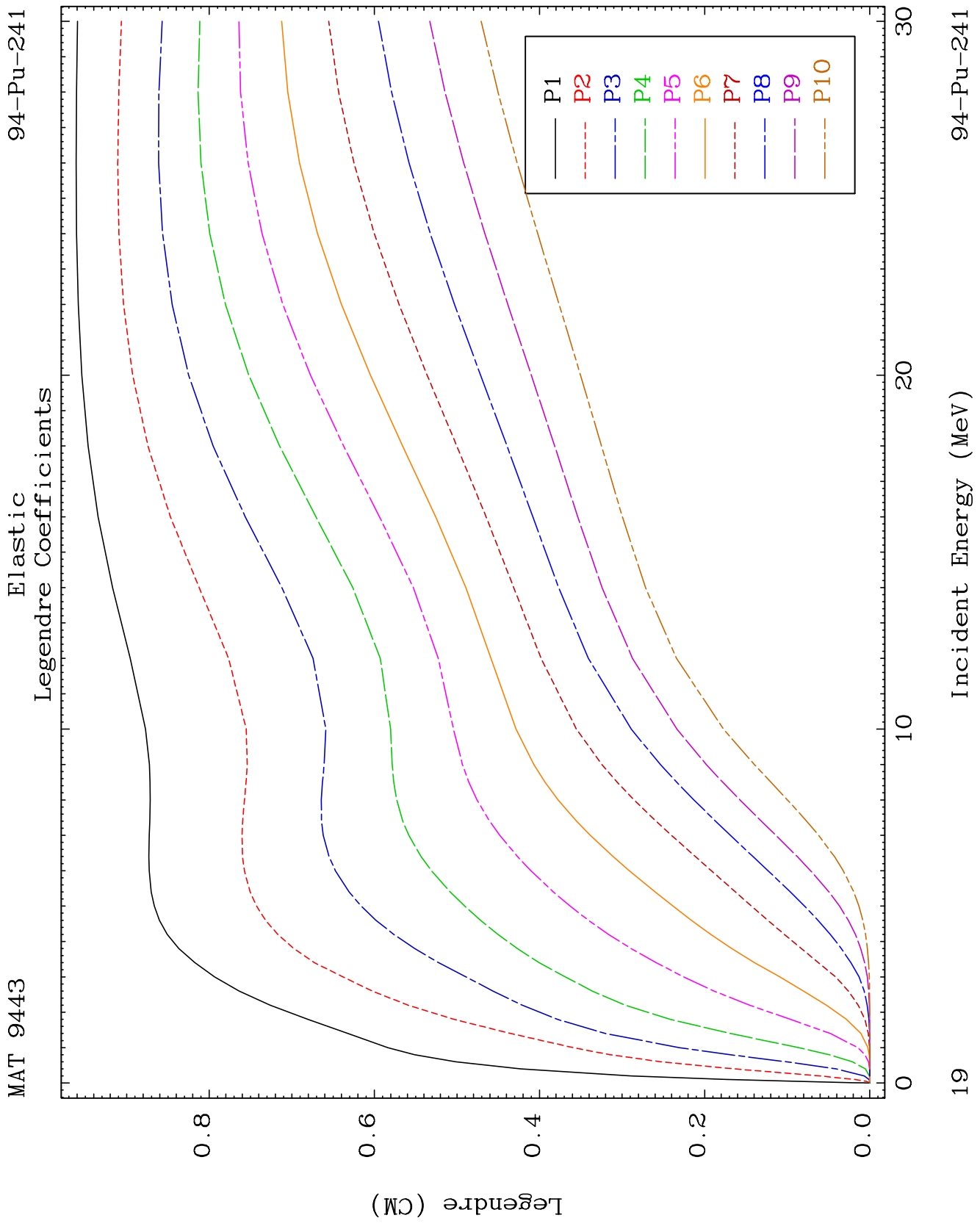
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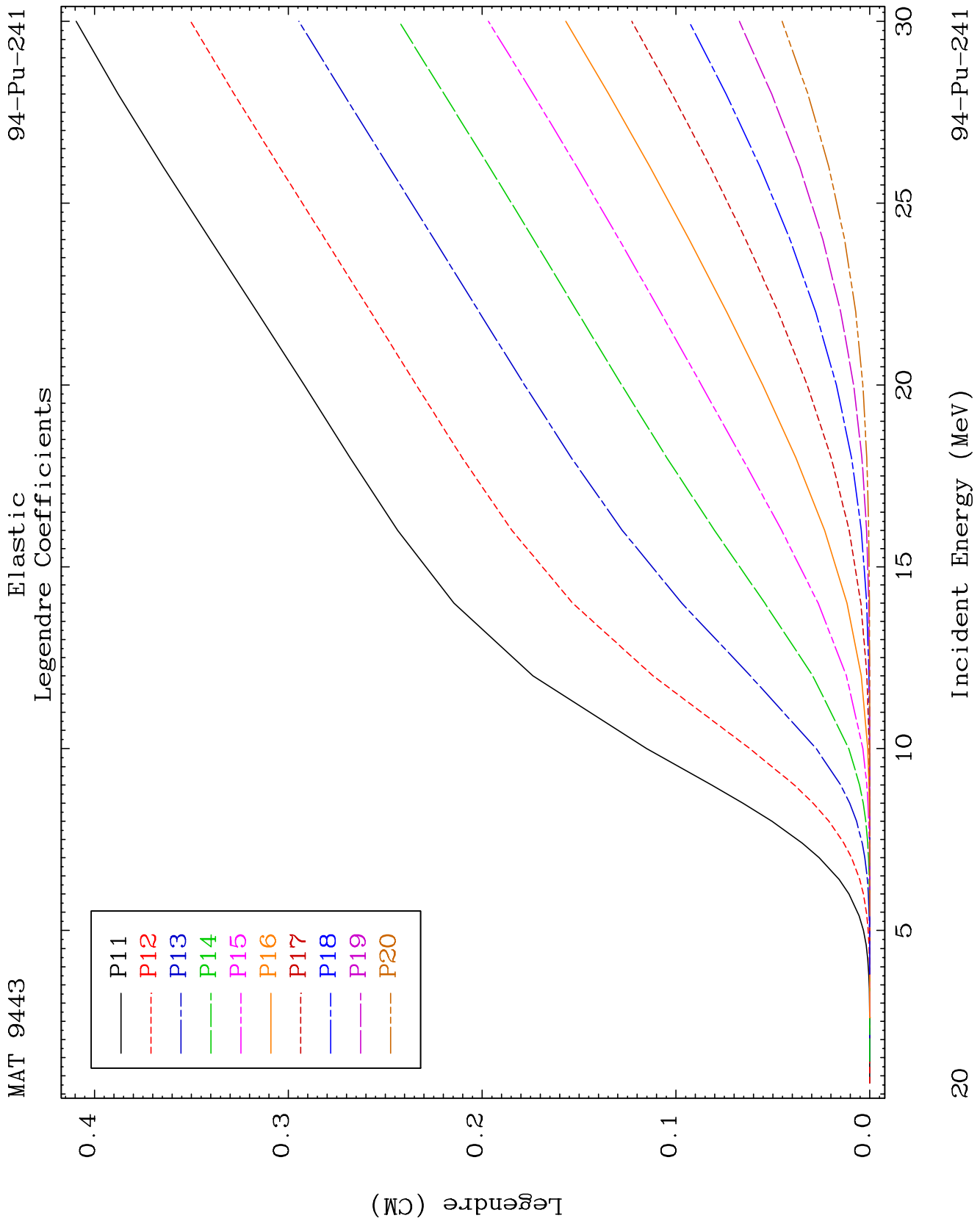


18

Incident Energy (MeV)

94-Pu-241

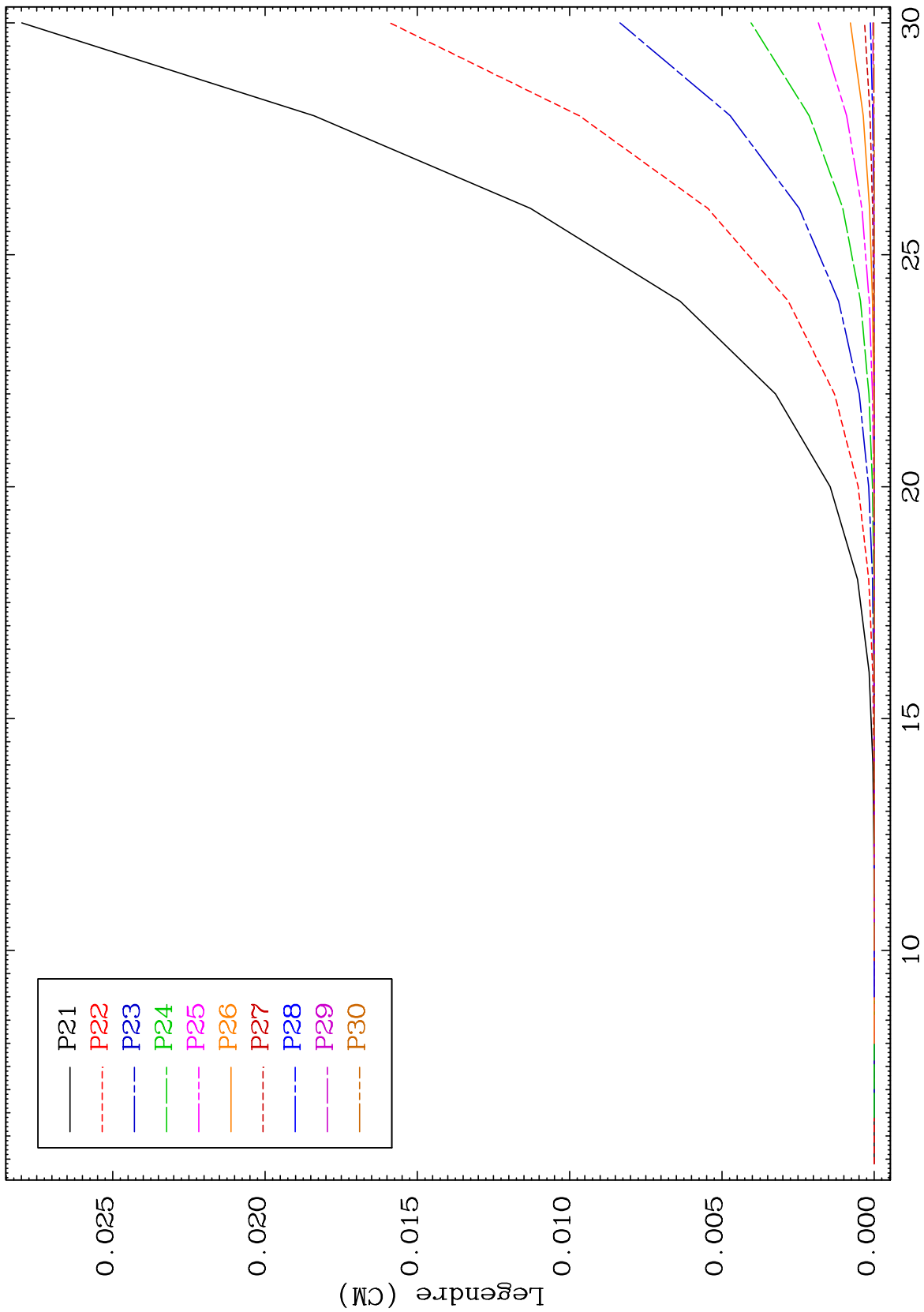




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### Elastic Legendre Coefficients

94-Pu-241



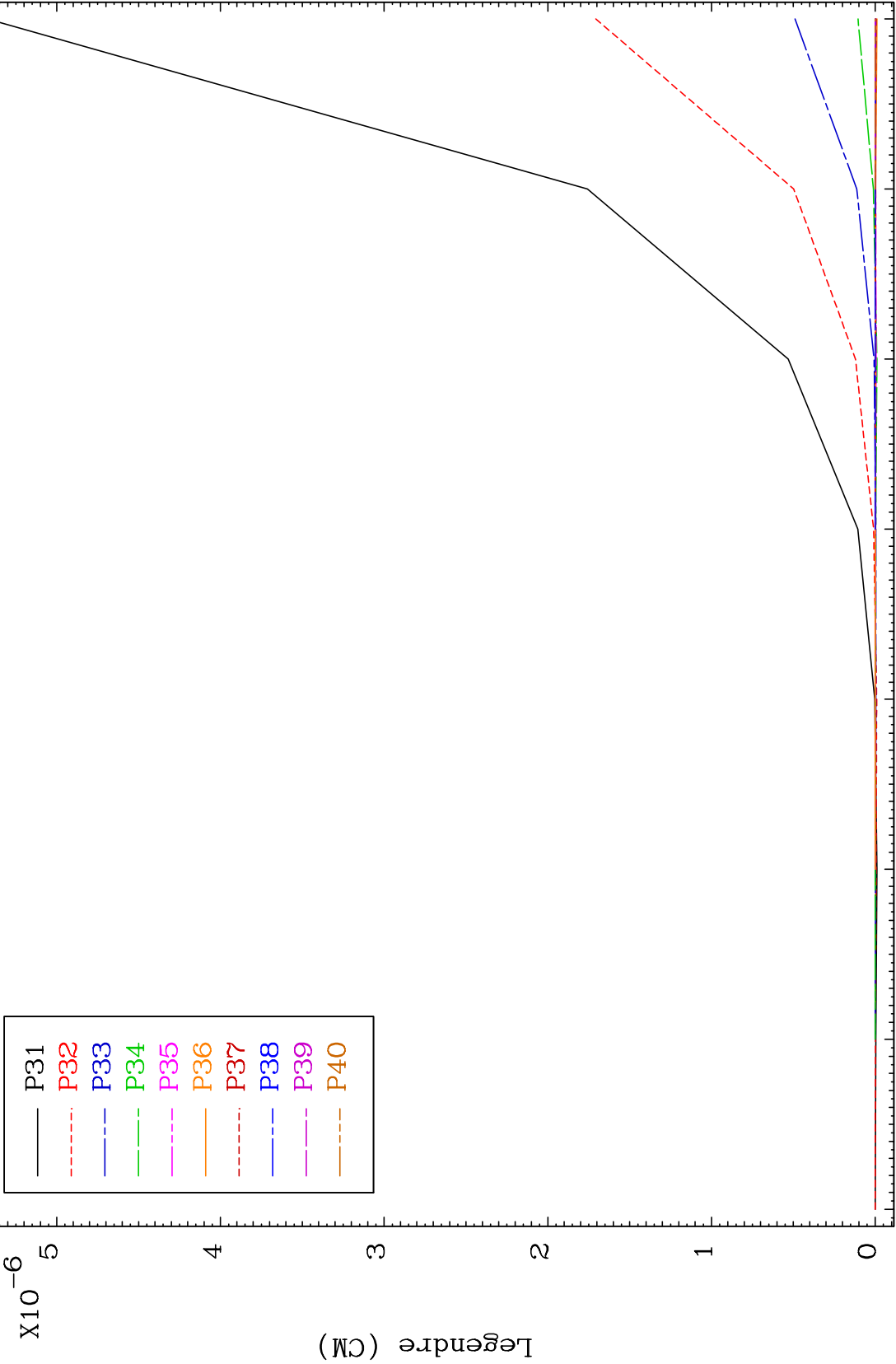
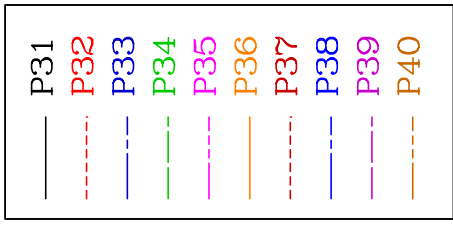
94-Pu-241

Incident Energy (MeV)

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Elastic Legendre Coefficients

94-Pu-241



94-Pu-241

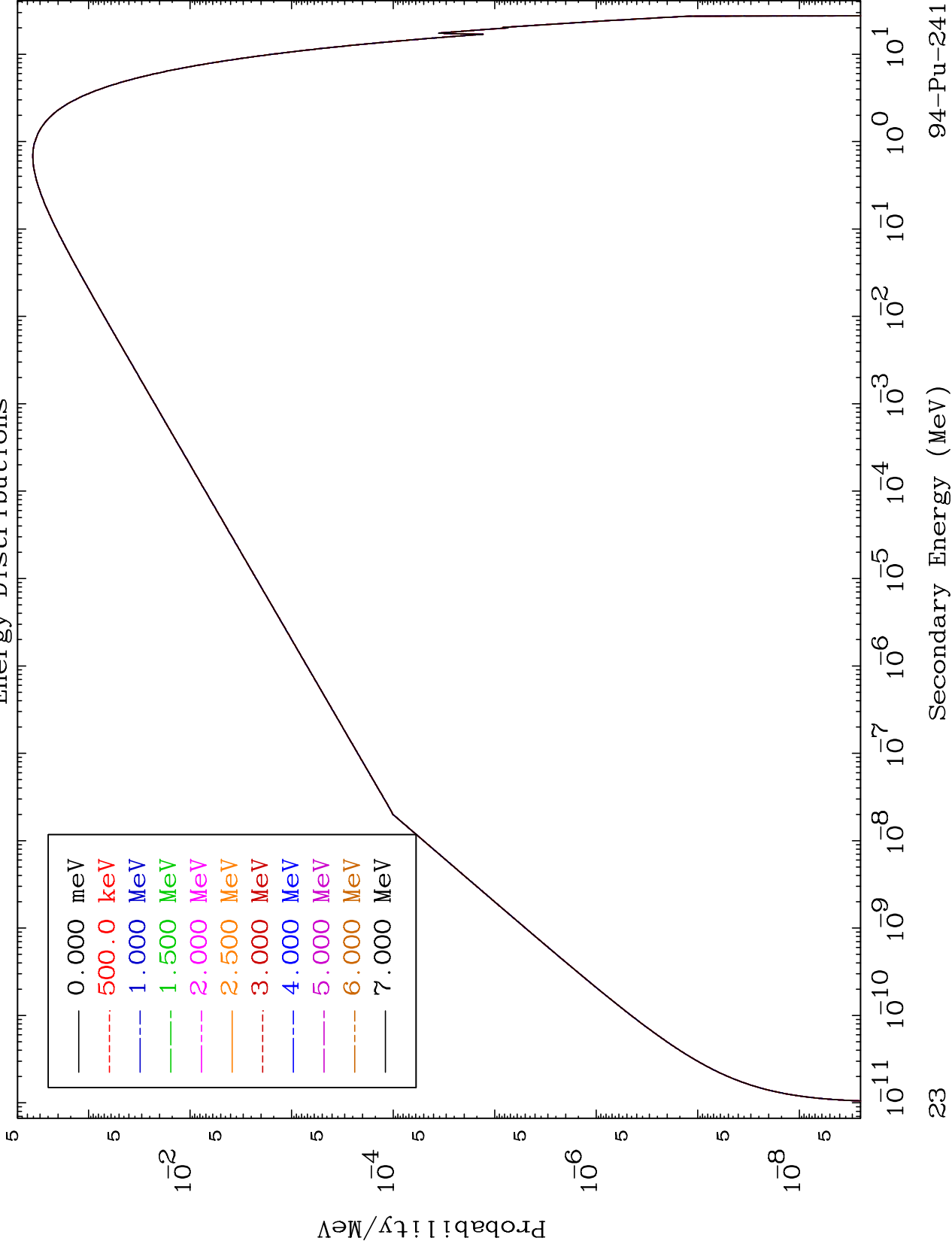
Incident Energy (MeV)

22

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### Fission Energy Distributions

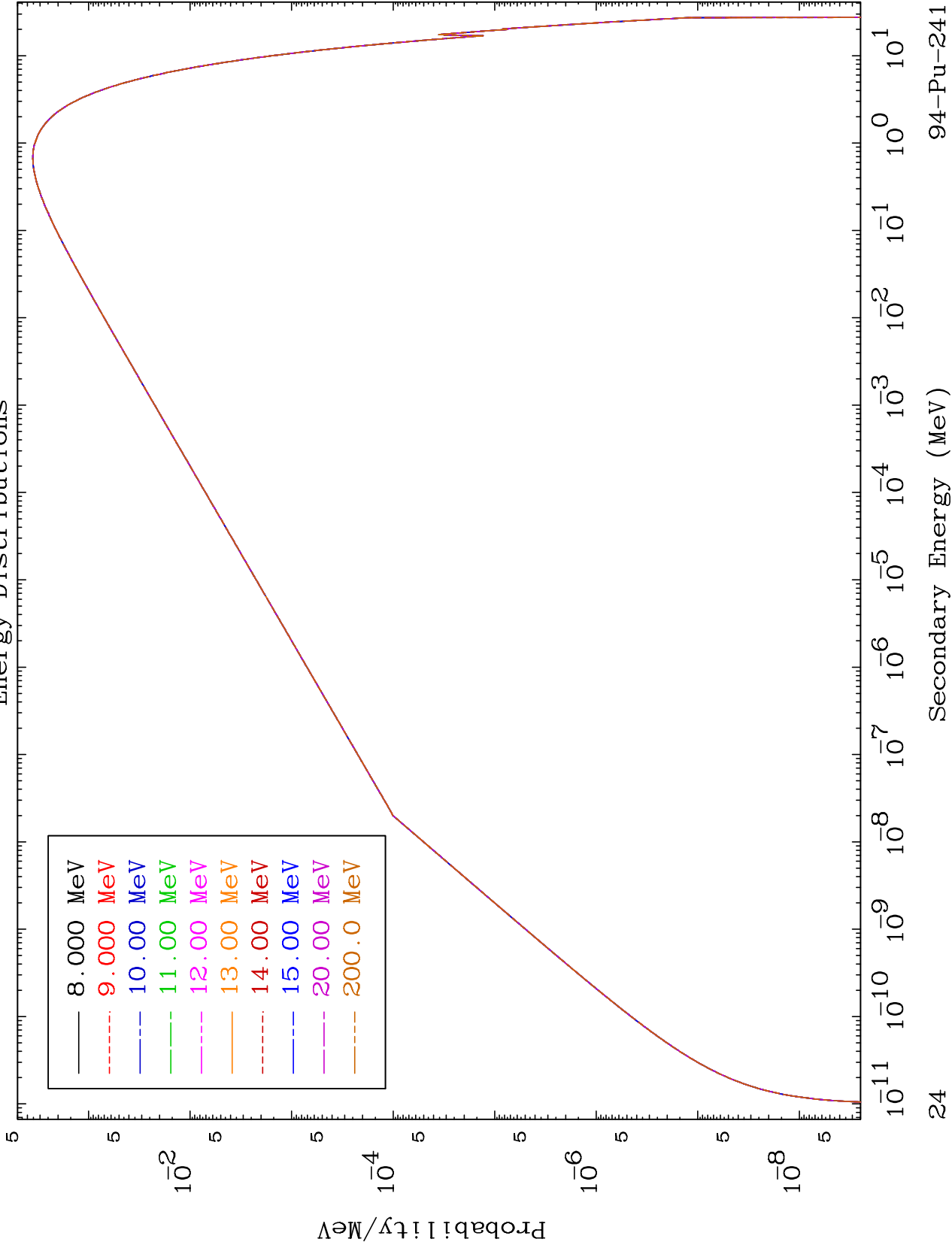
94-Pu-241



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### Fission Energy Distributions

94-Pu-241



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Secondary Energy (MeV)

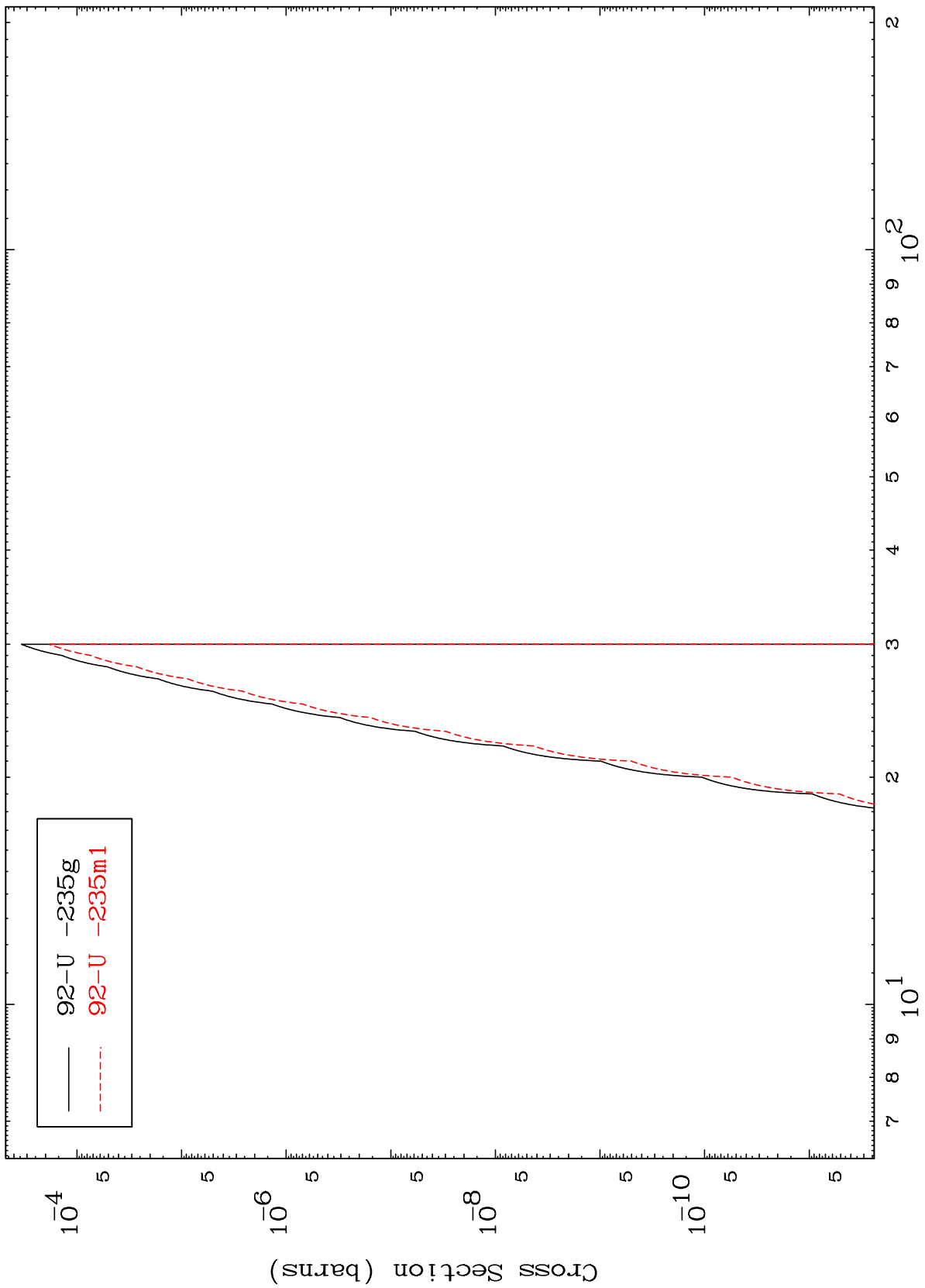
24

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(n,3n)  $\alpha$

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Radionuclide Production Cross Section



25

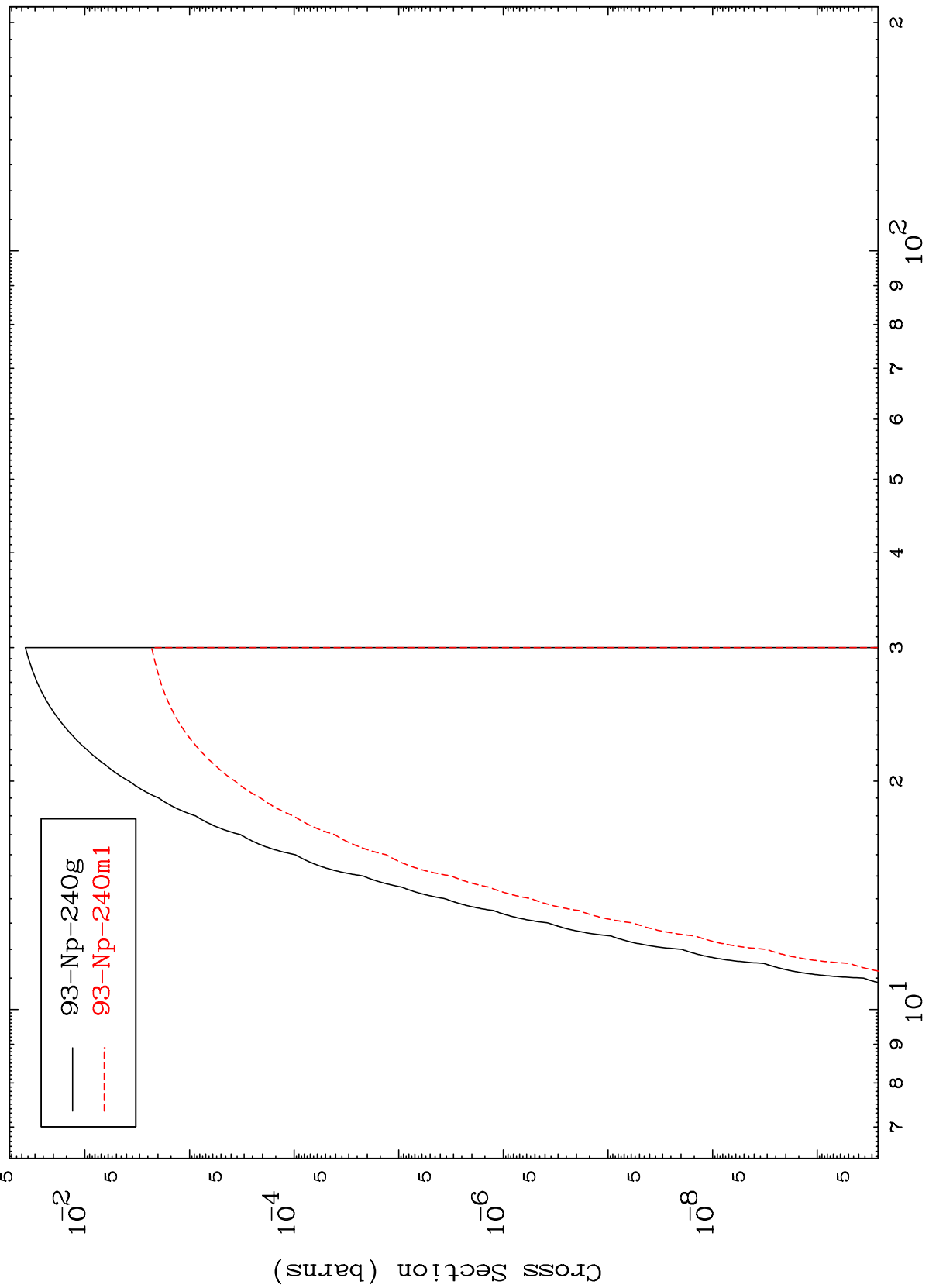
Incident Energy (MeV)

94-Pu-241

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94-Pu-241

(n,n') p  
Radionuclide Production Cross Section



26

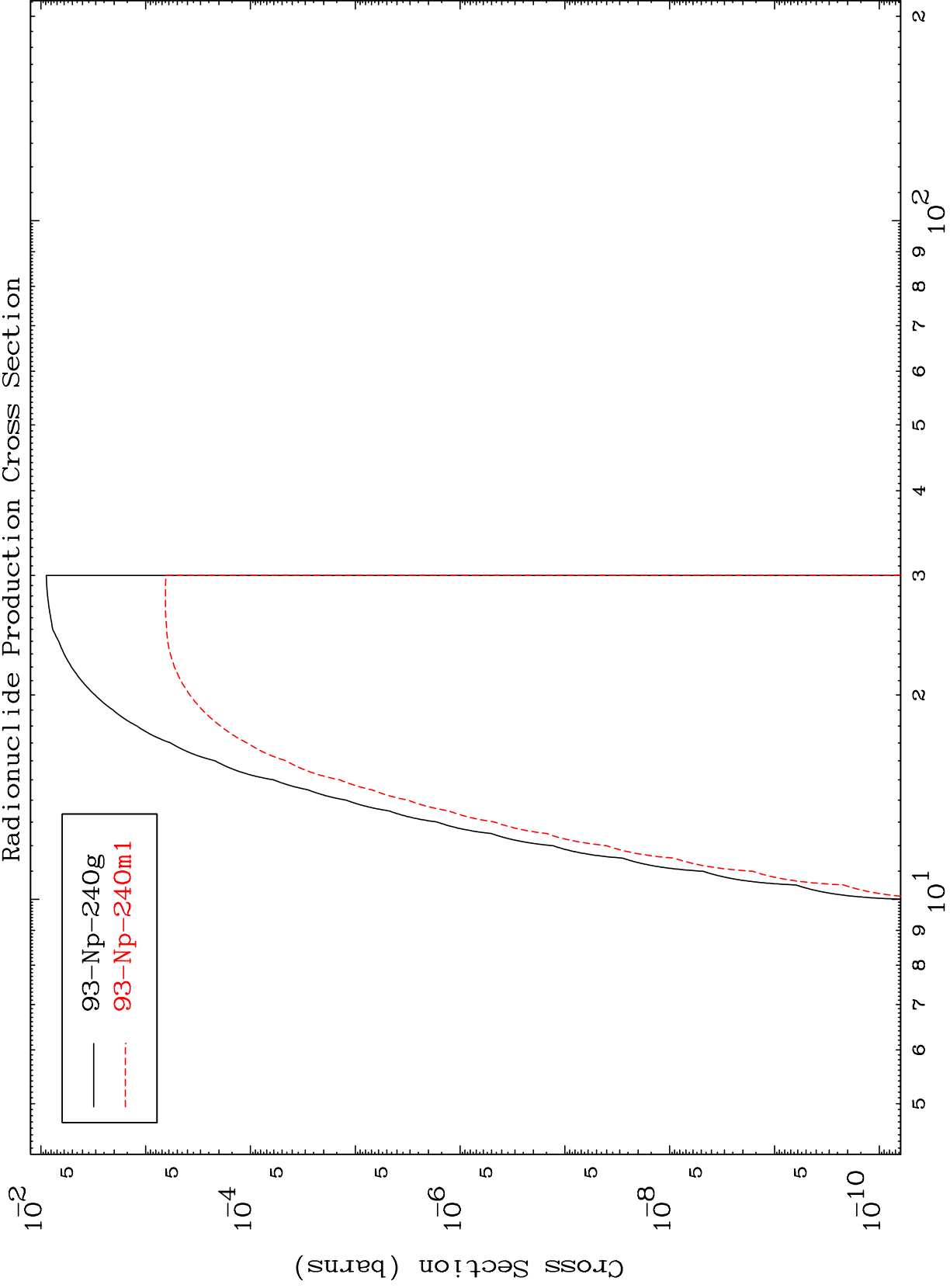
Incident Energy (MeV)

94-Pu-241

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94-Pu-241

(n,d)  
Radionuclide Production Cross Section



94-Pu-241

Incident Energy (MeV)

27