

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

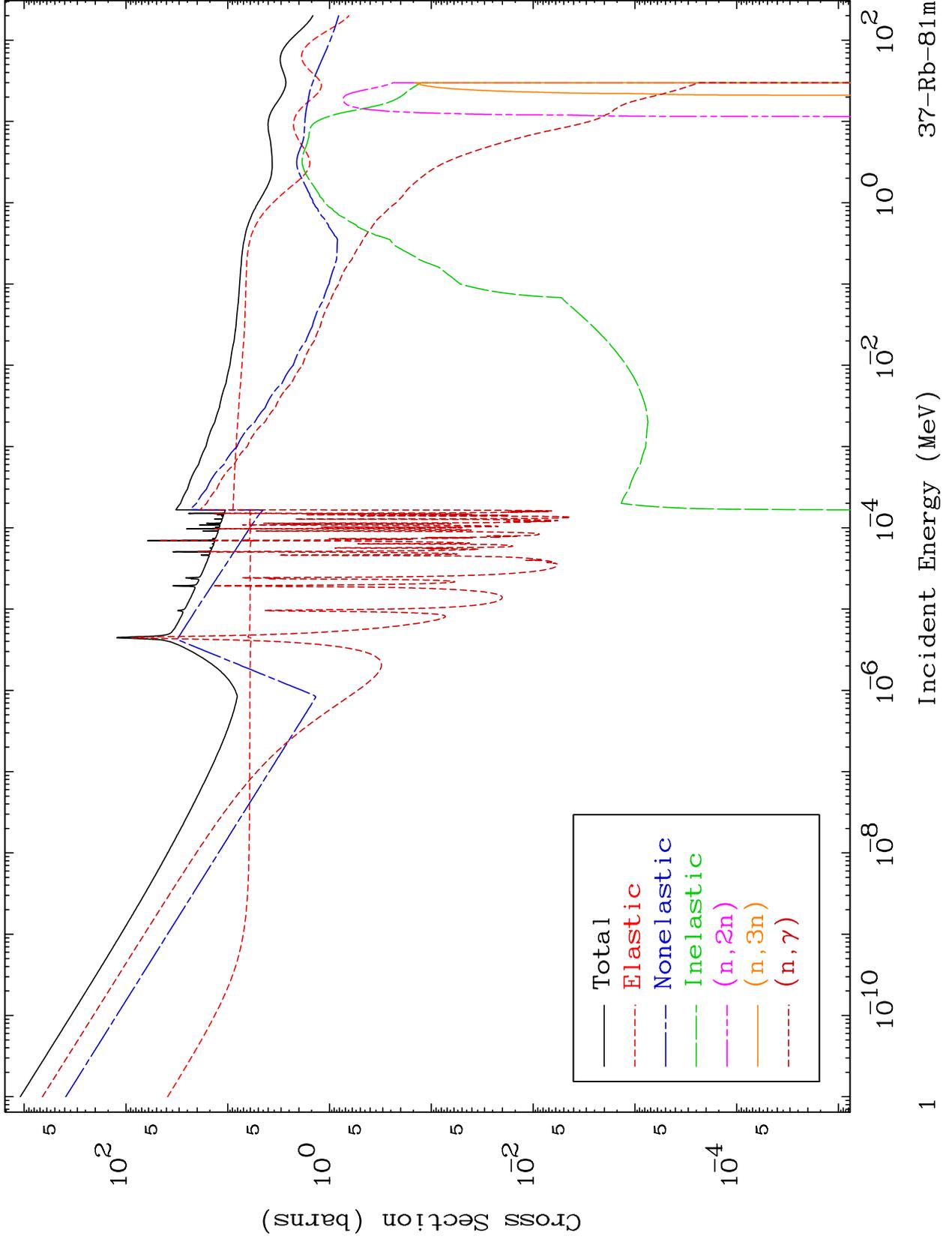
Web:redcullen1.net/HOMEPAGE.NEW

Press Mouse Button to Start

MAT 3714

Neutron Major
293 Kelvin Cross Sections

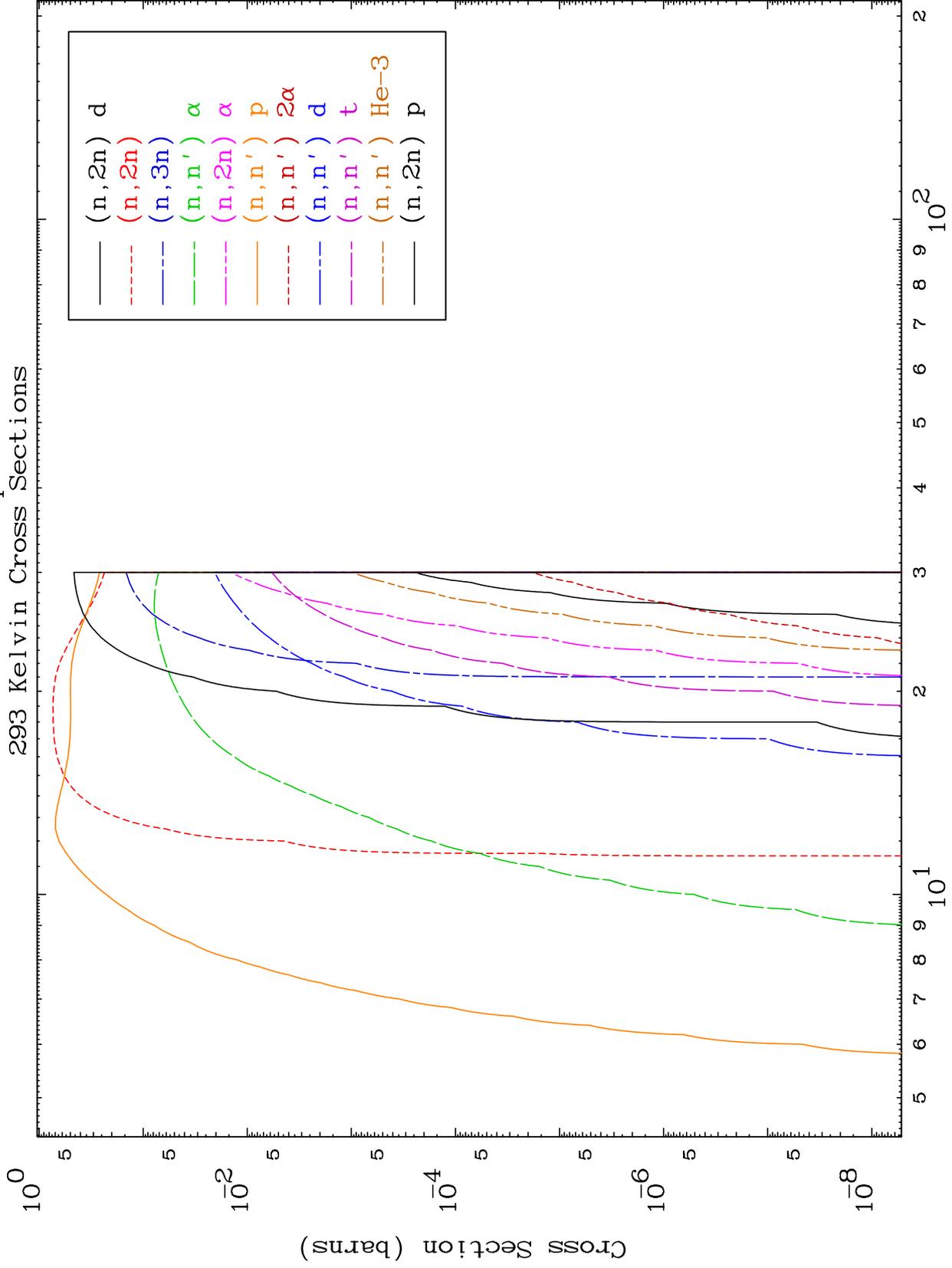
37-Rb-81m



MAT 3714

Neutron Absorption
293 Kelvin Cross Sections

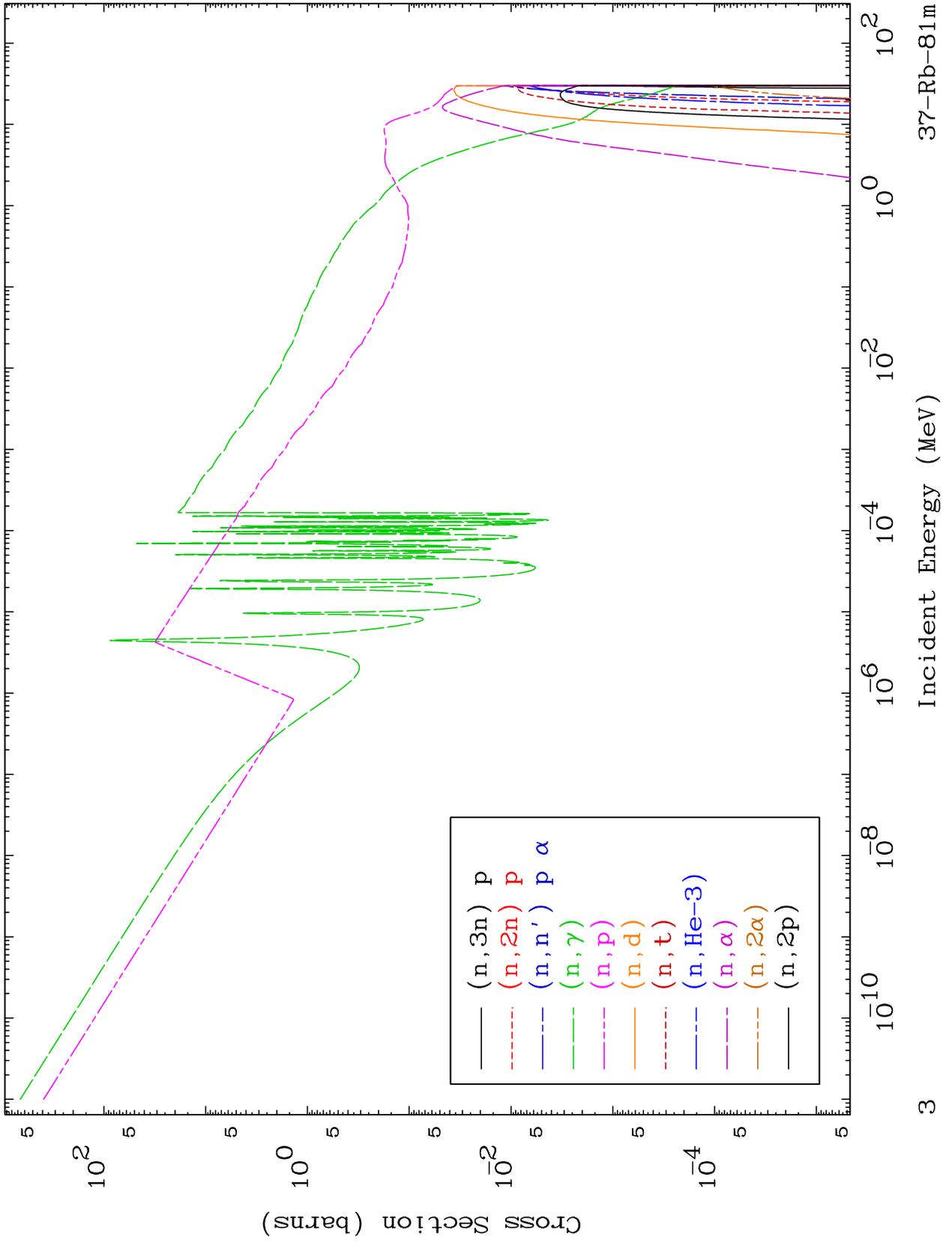
37-Rb-81m



MAT 3714

Neutron Absorption
293 Kelvin Cross Sections

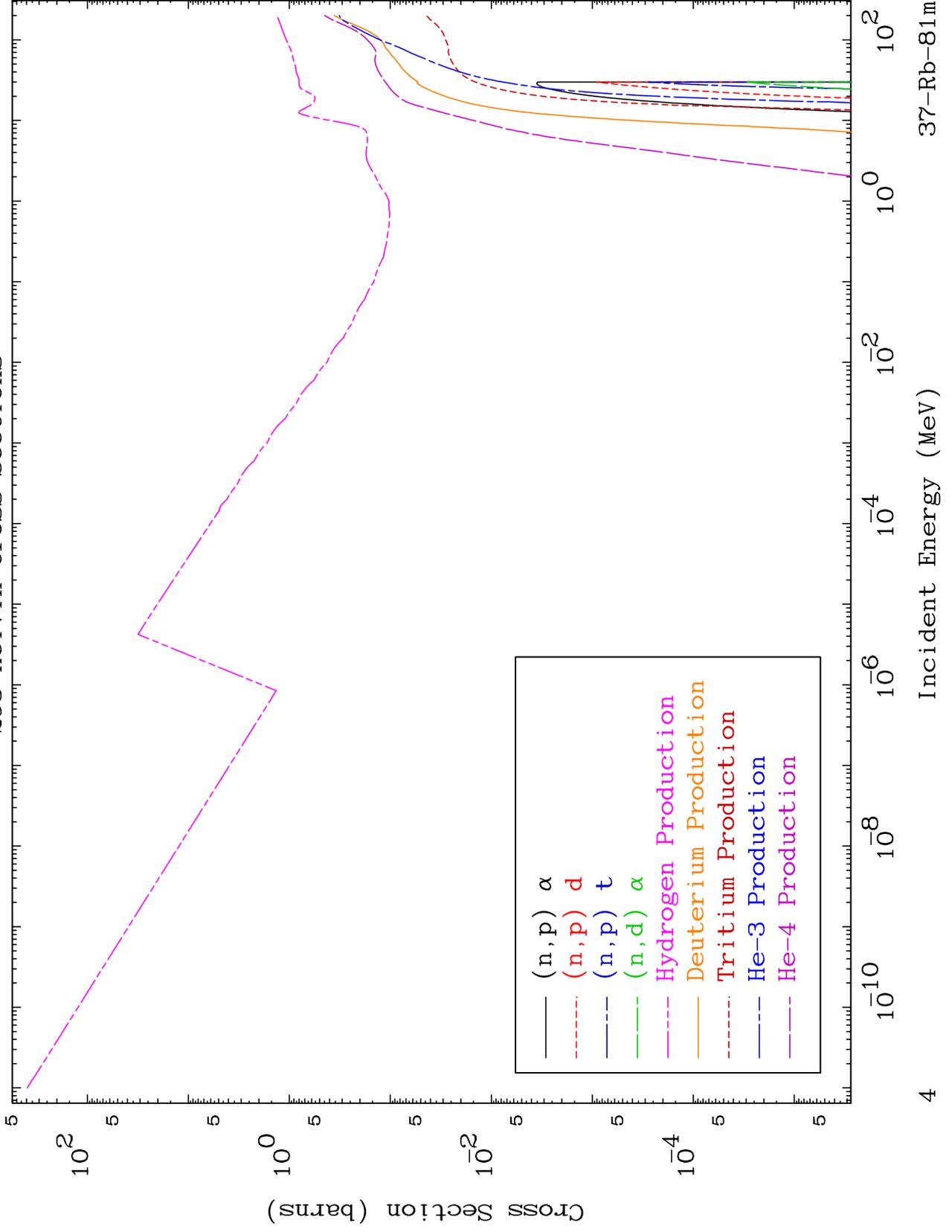
37-Rb-81m



MAT 3714

Neutron Absorption
293 Kelvin Cross Sections

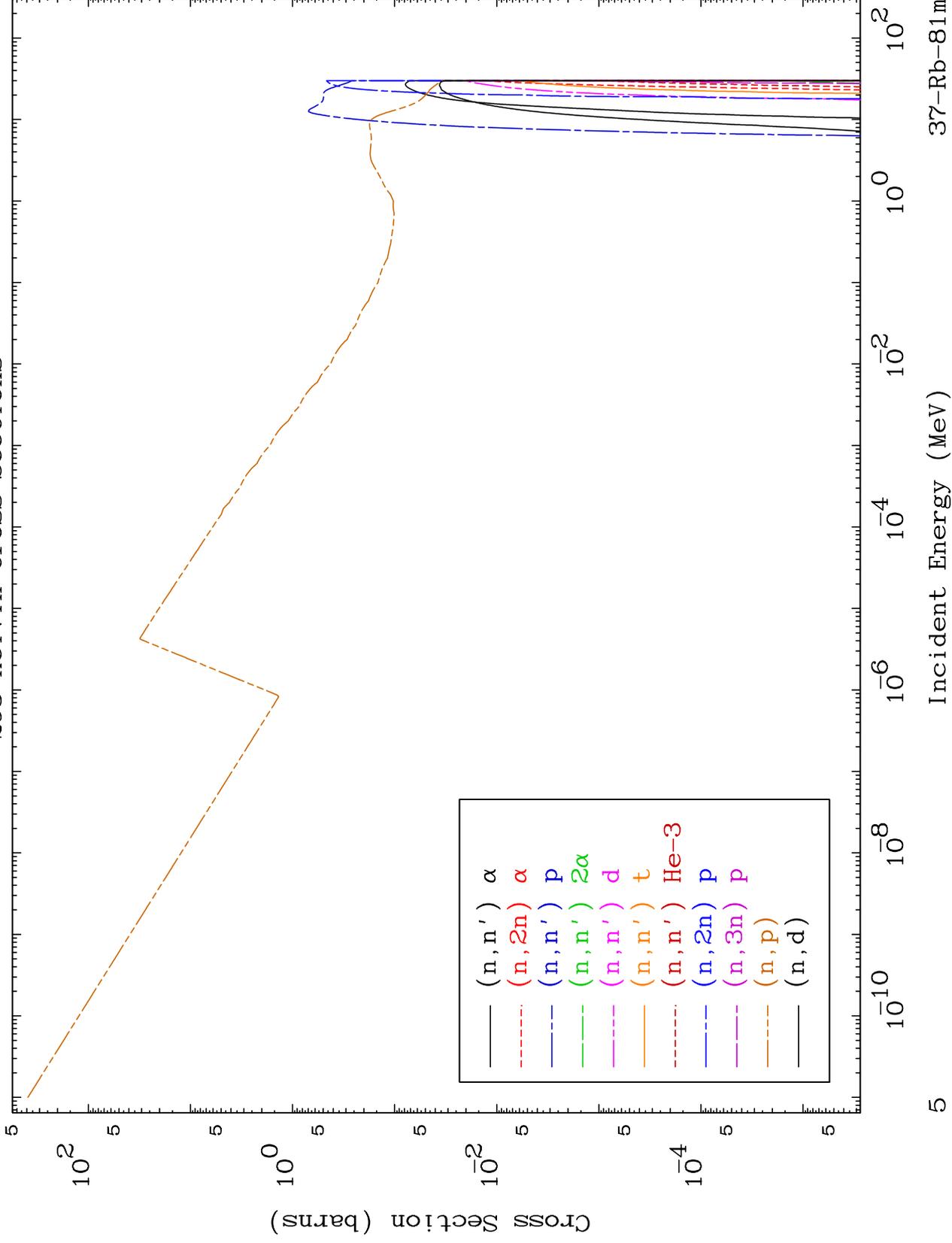
37-Rb-81m



MAT 3714

Charged Particle
293 Kelvin Cross Sections

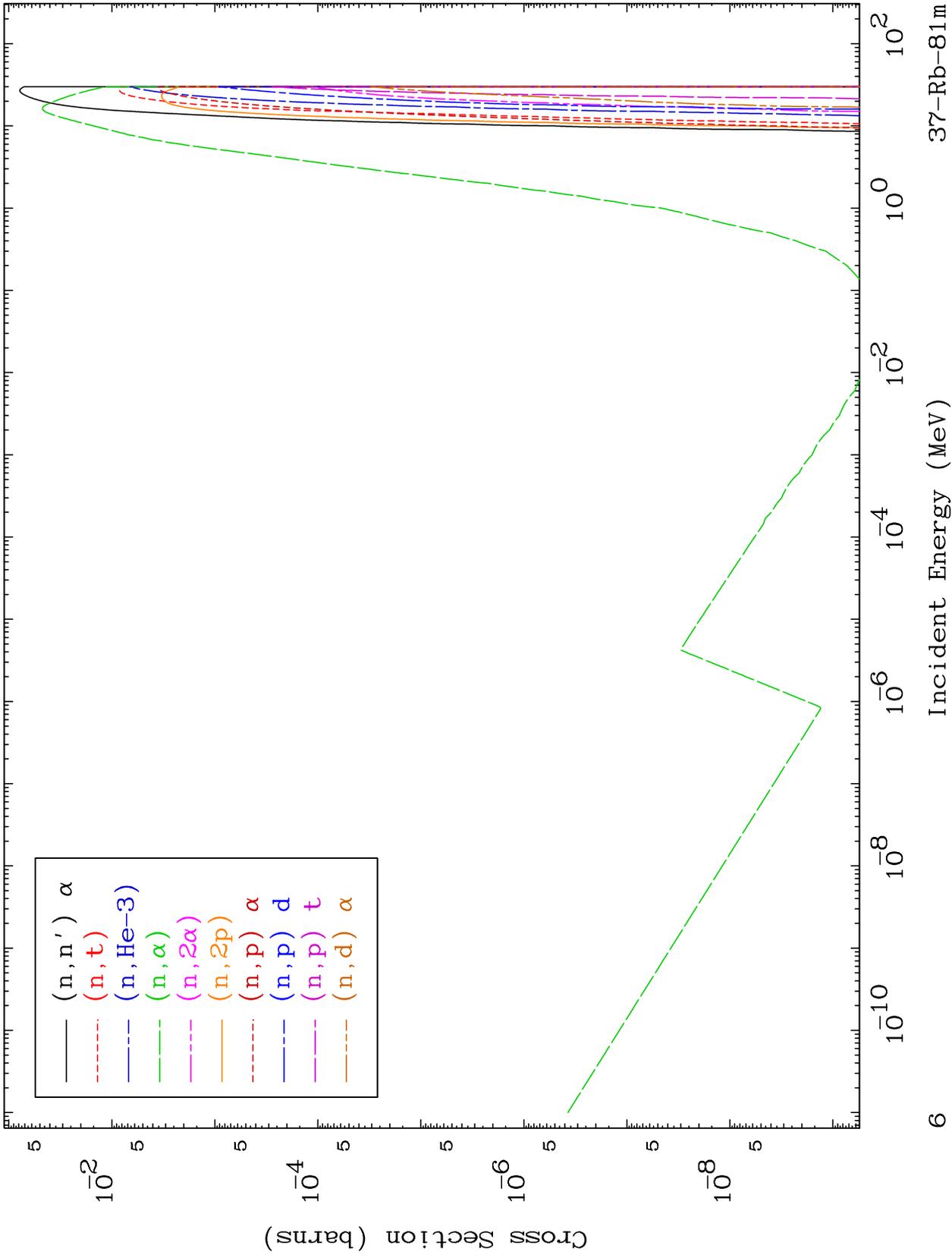
37-Rb-81m



MAT 3714

Charged Particle
293 Kelvin Cross Sections

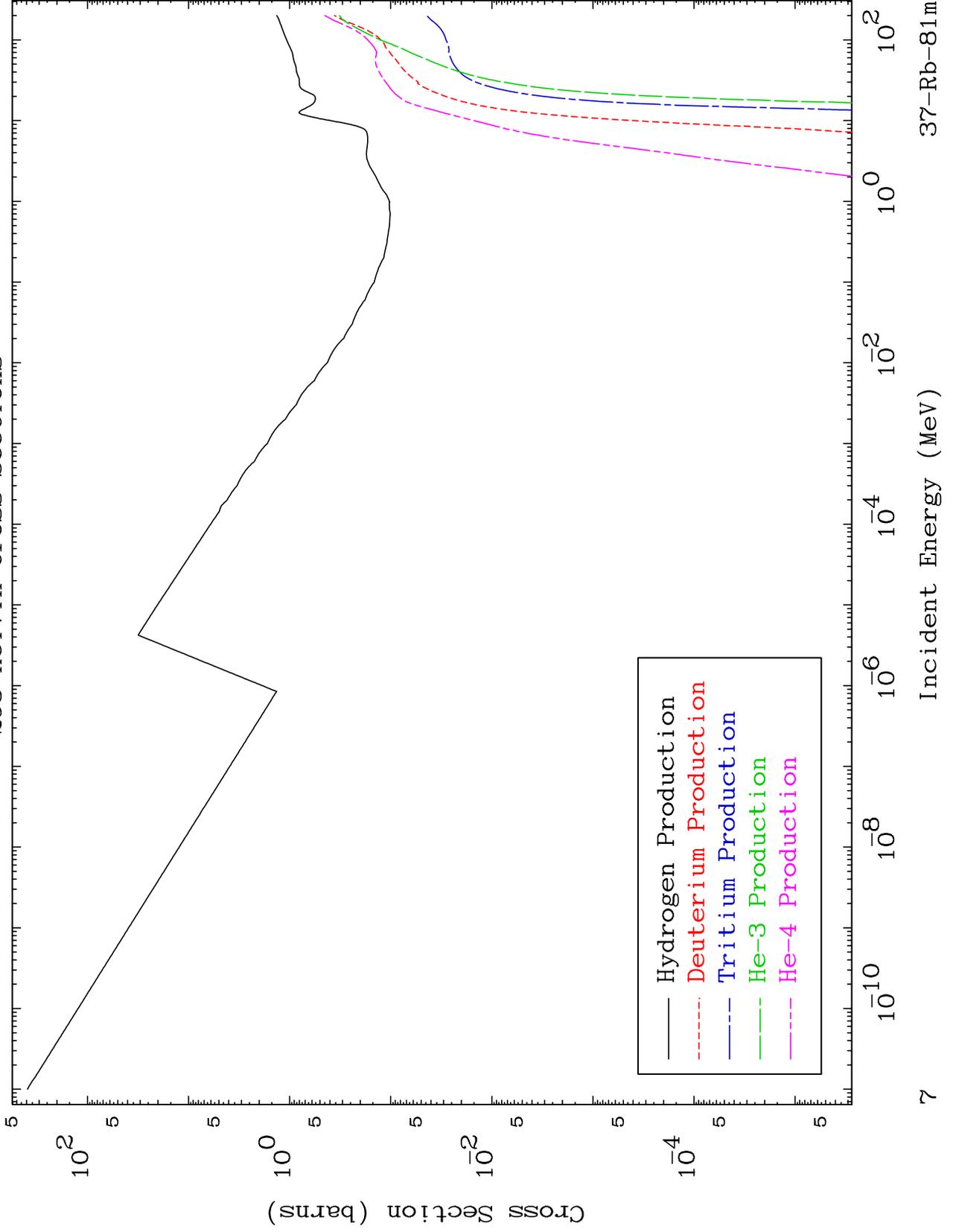
37-Rb-81m



MAT 3714

Particle Production
293 Kelvin Cross Sections

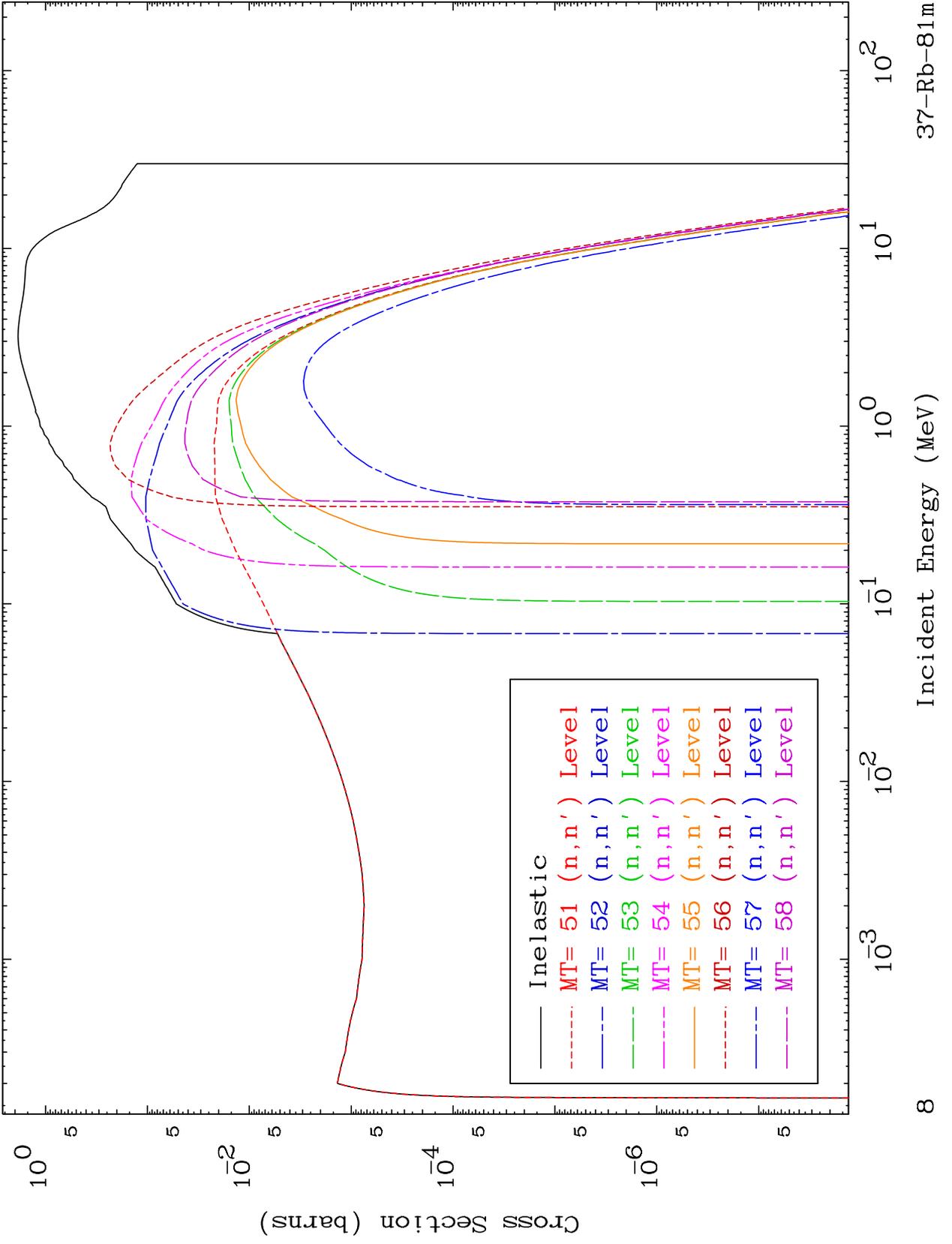
37-Rb-81m



MAT 3714

(n,n') Levels
293 Kelvin Cross Sections

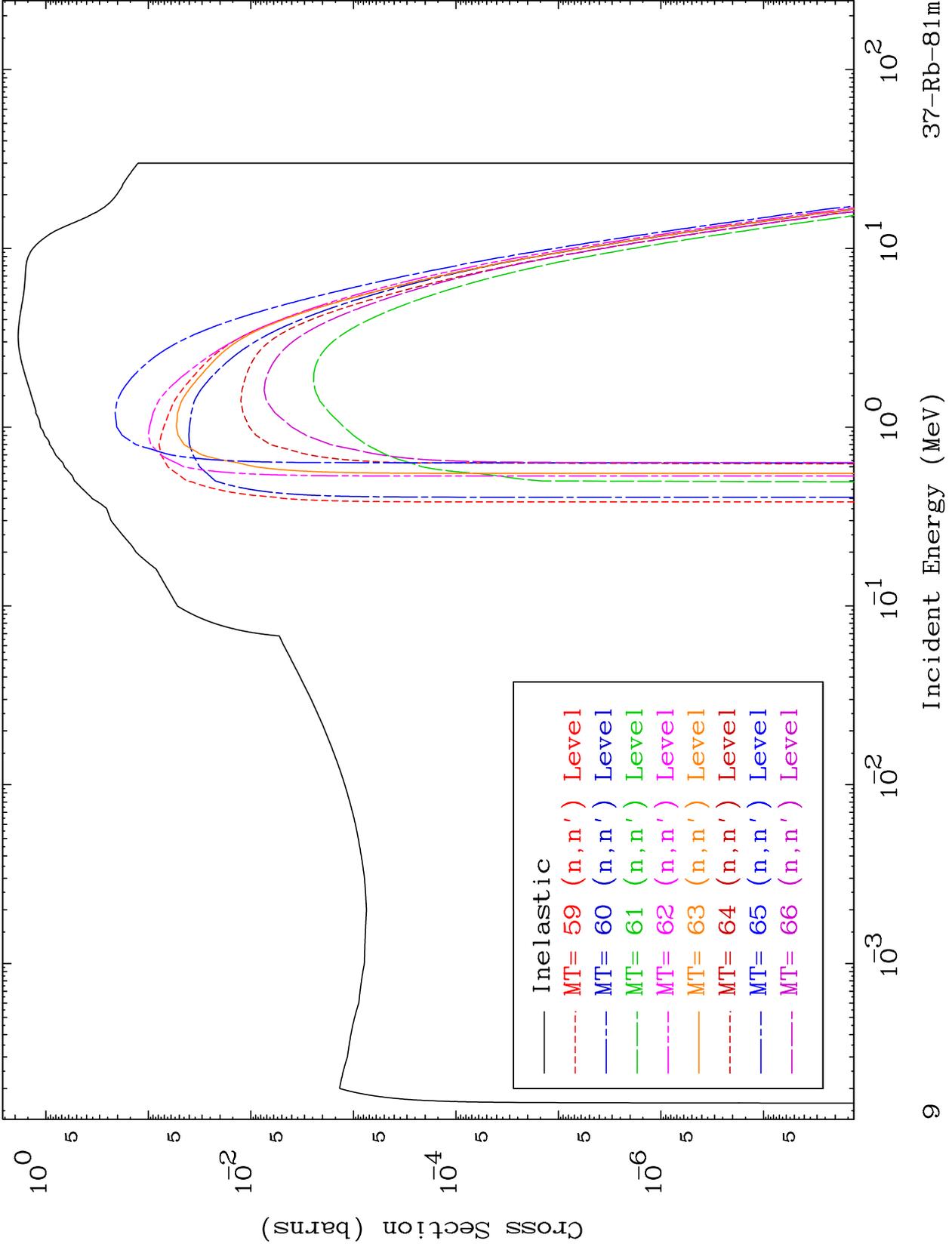
37-Rb-81m



MAT 3714

(n,n') Levels
293 Kelvin Cross Sections

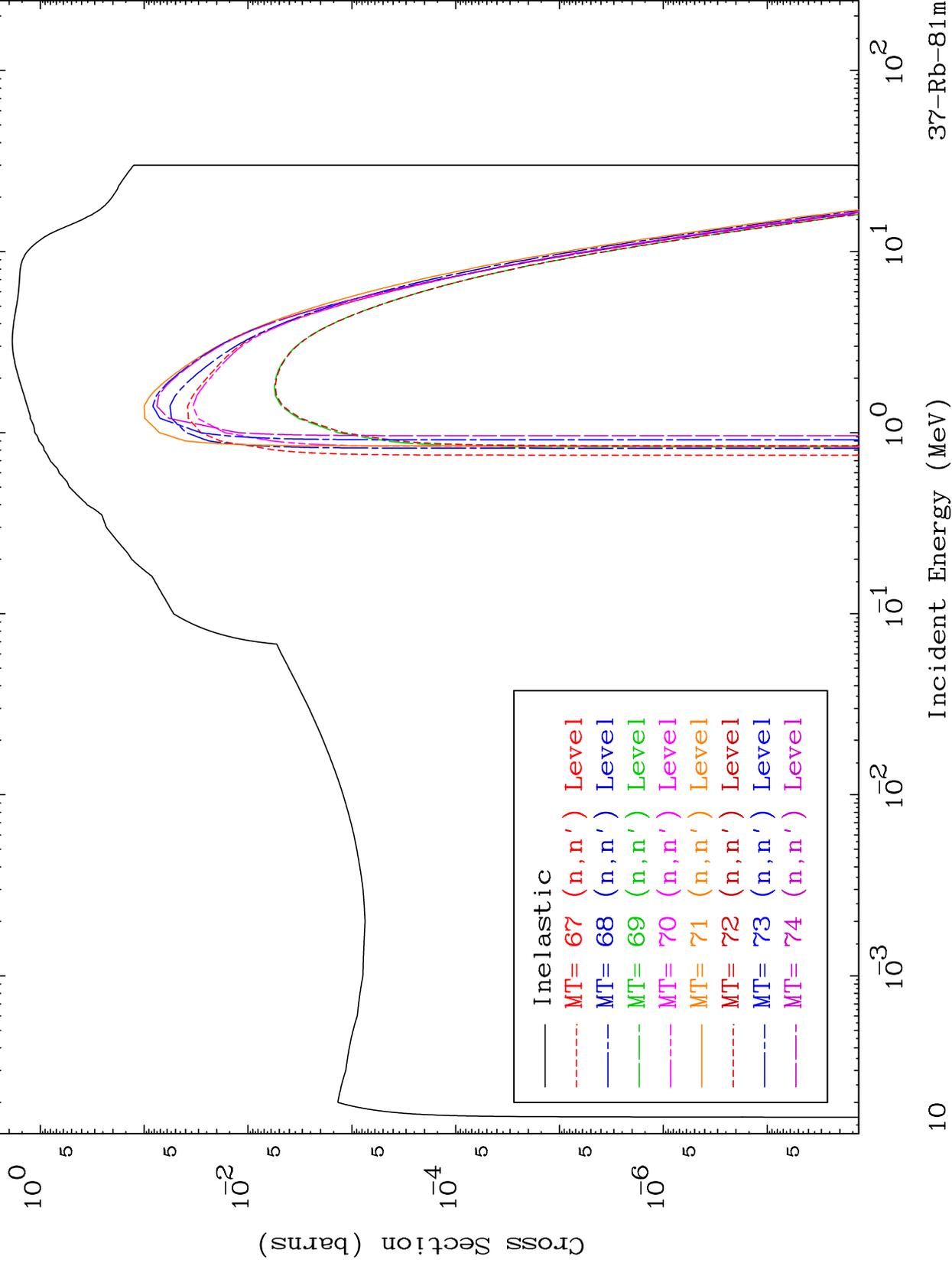
37-Rb-81m



MAT 3714

(n,n') Levels
293 Kelvin Cross Sections

37-Rb-81m

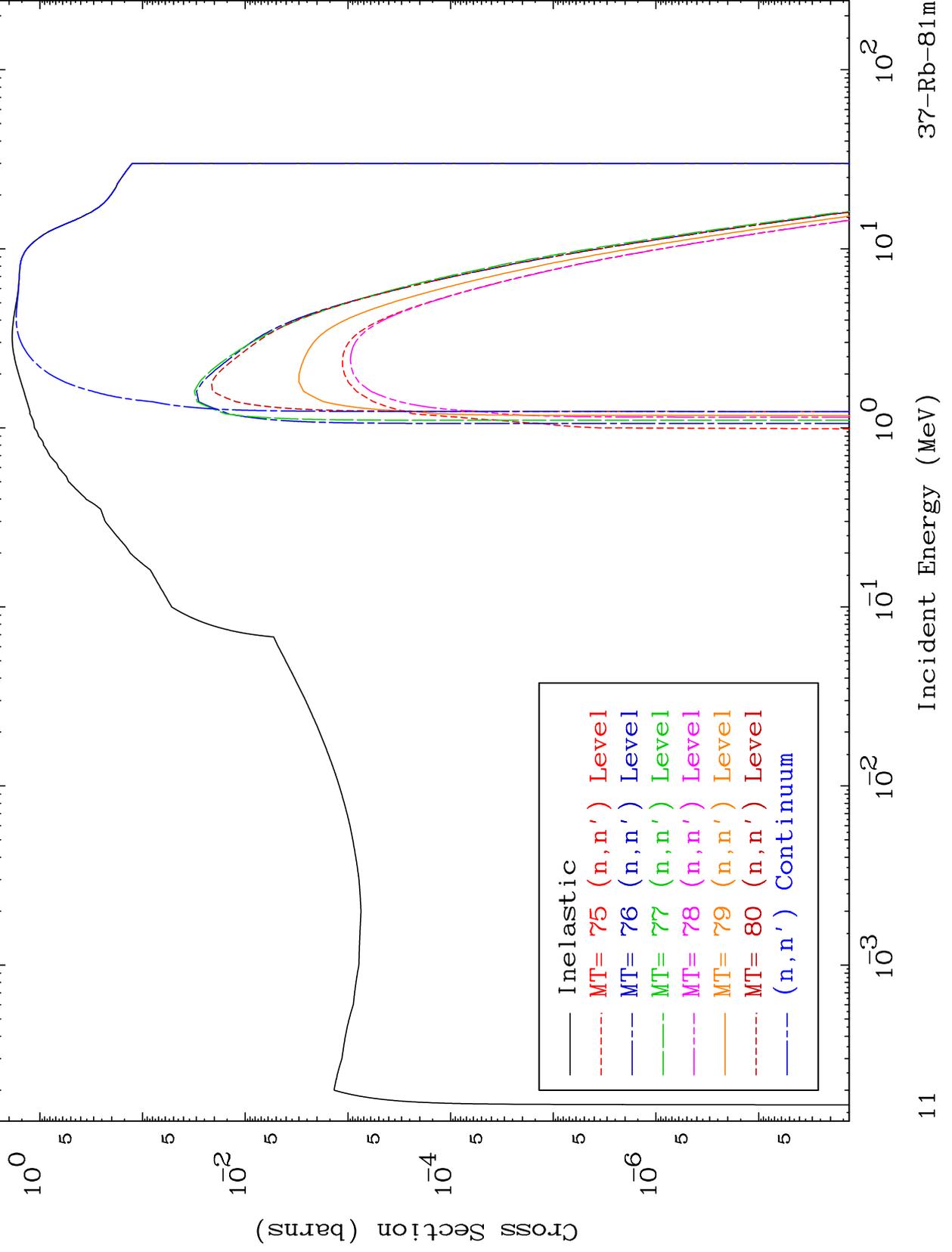


10

MAT 3714

(n,n') Levels
293 Kelvin Cross Sections

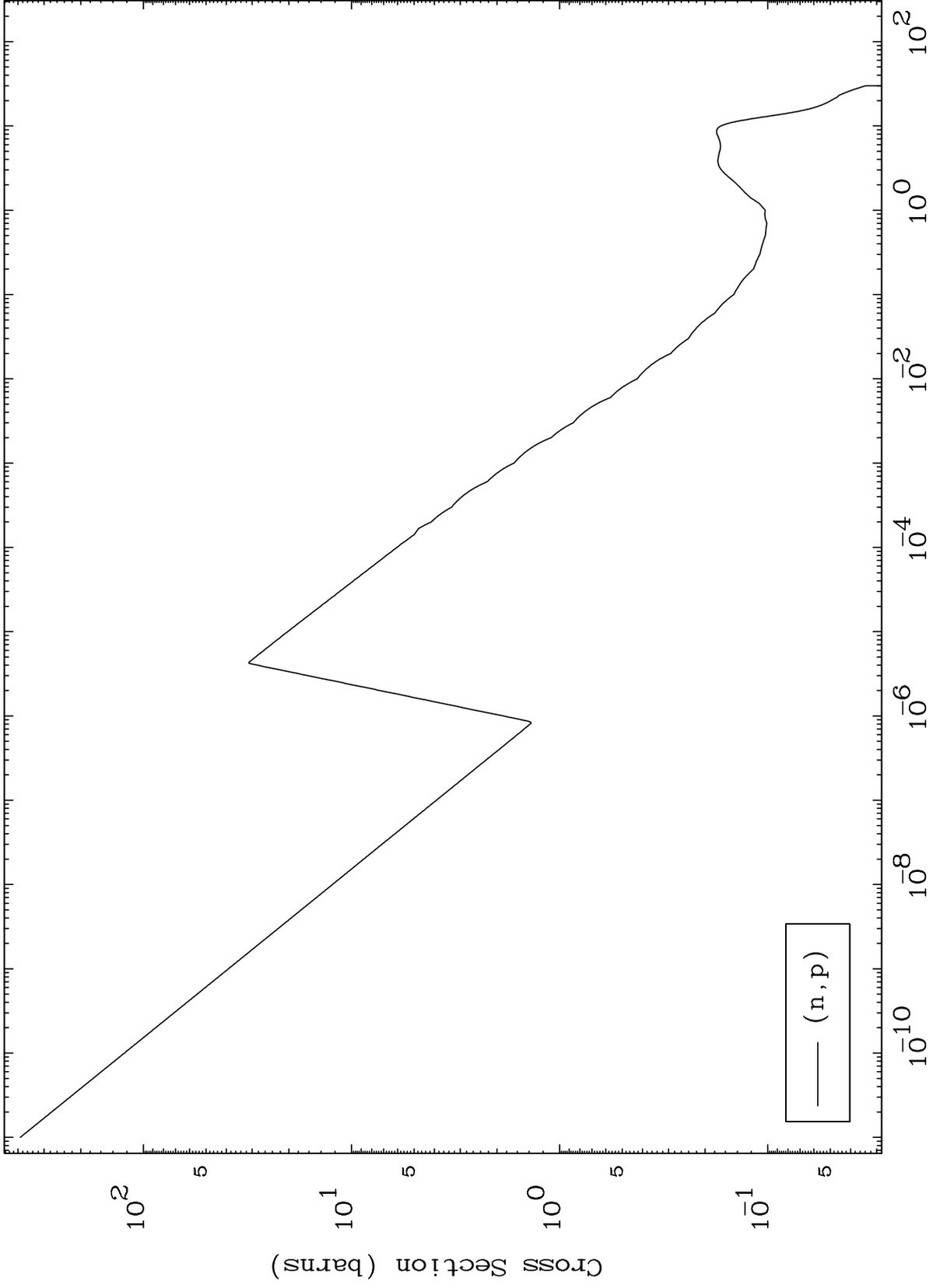
37-Rb-81m



MAT 3714

(n,p) Levels
293 Kelvin Cross Sections

37-Rb-81m

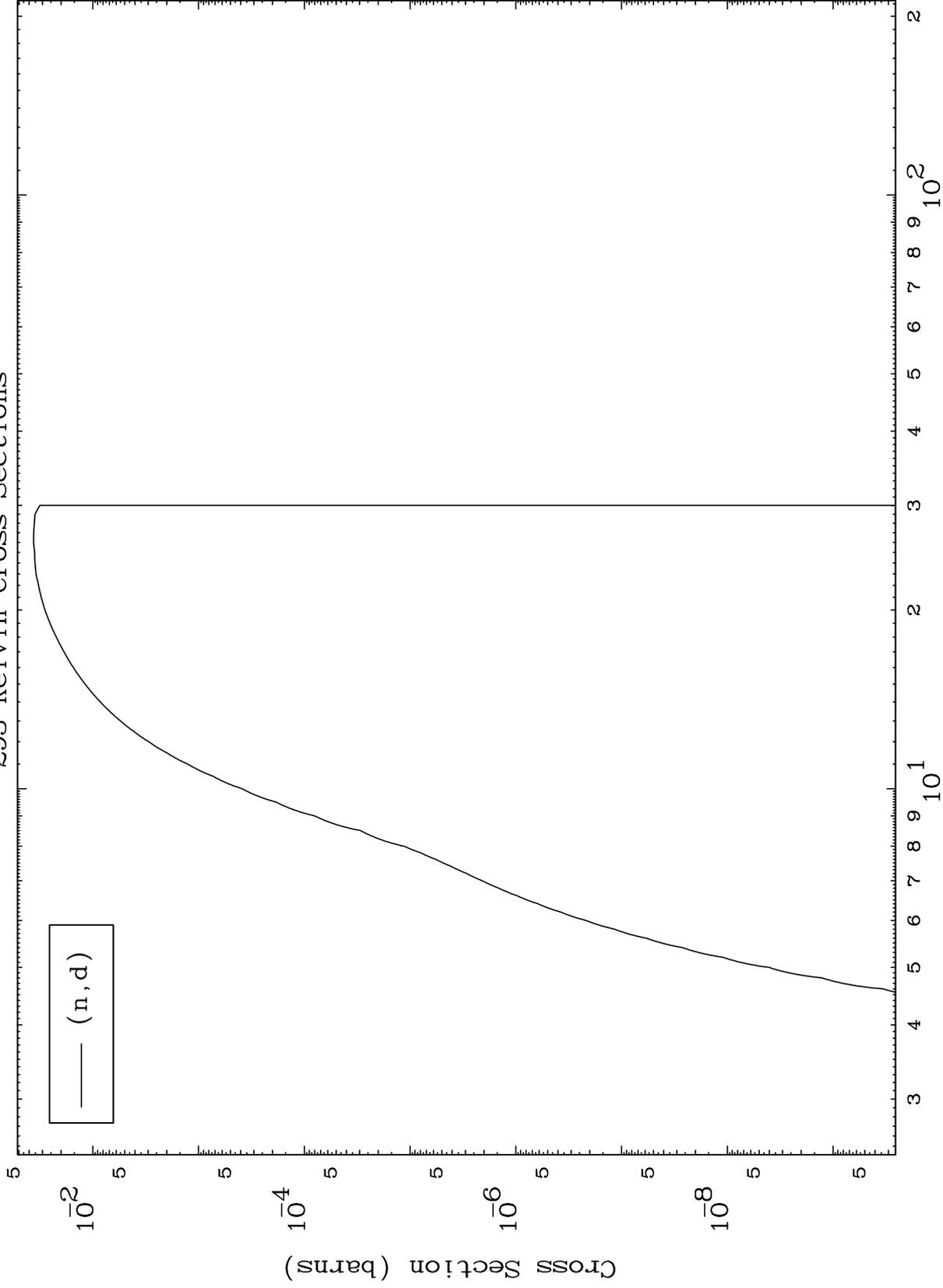


(n,p)

MAT 3714

(n,d) Levels
293 Kelvin Cross Sections

37-Rb-81m



(n,d)

Incident Energy (MeV)

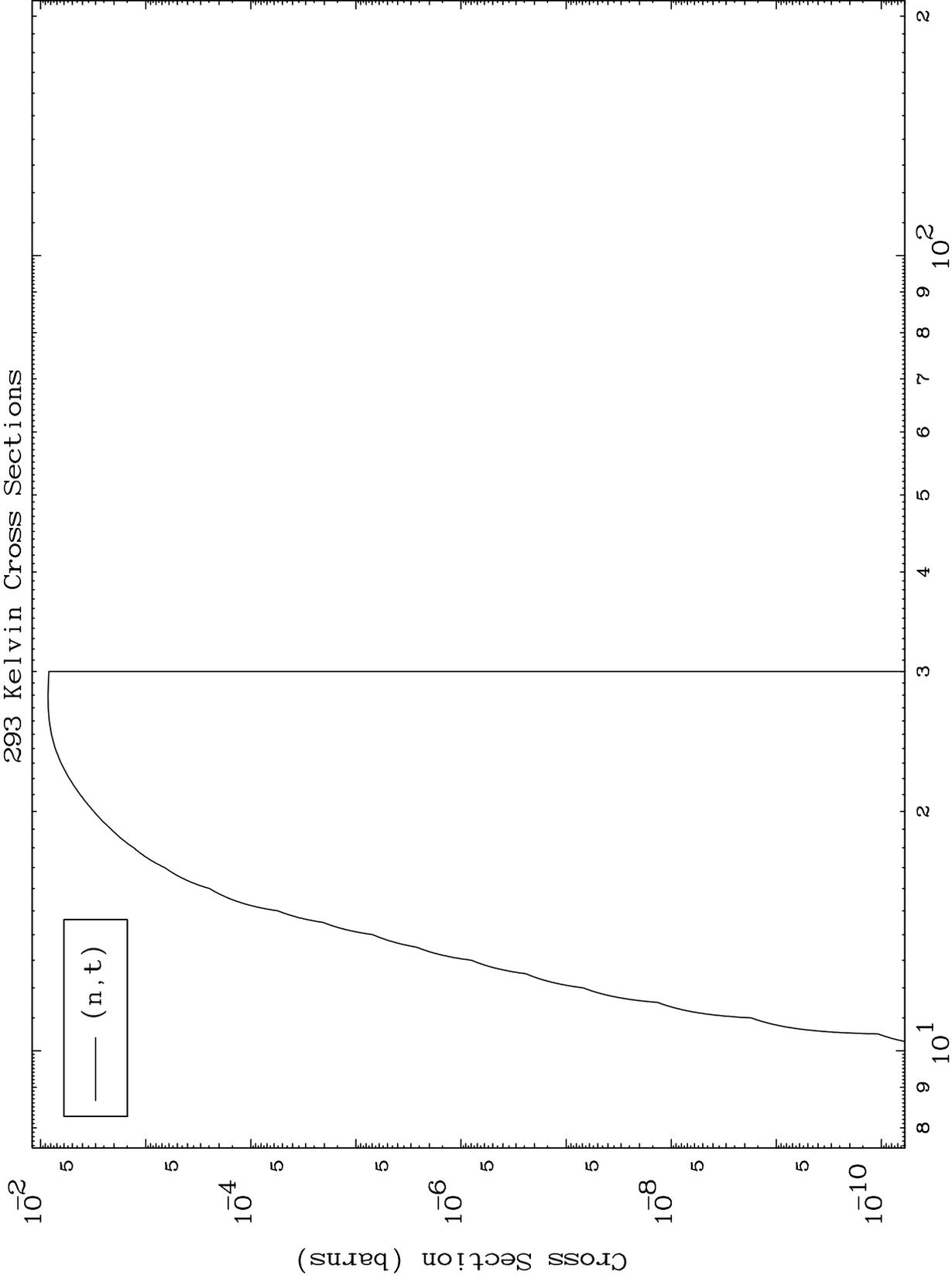
37-Rb-81m

13

MAT 3714

(n,t) Levels
293 Kelvin Cross Sections

37-Rb-81m



14

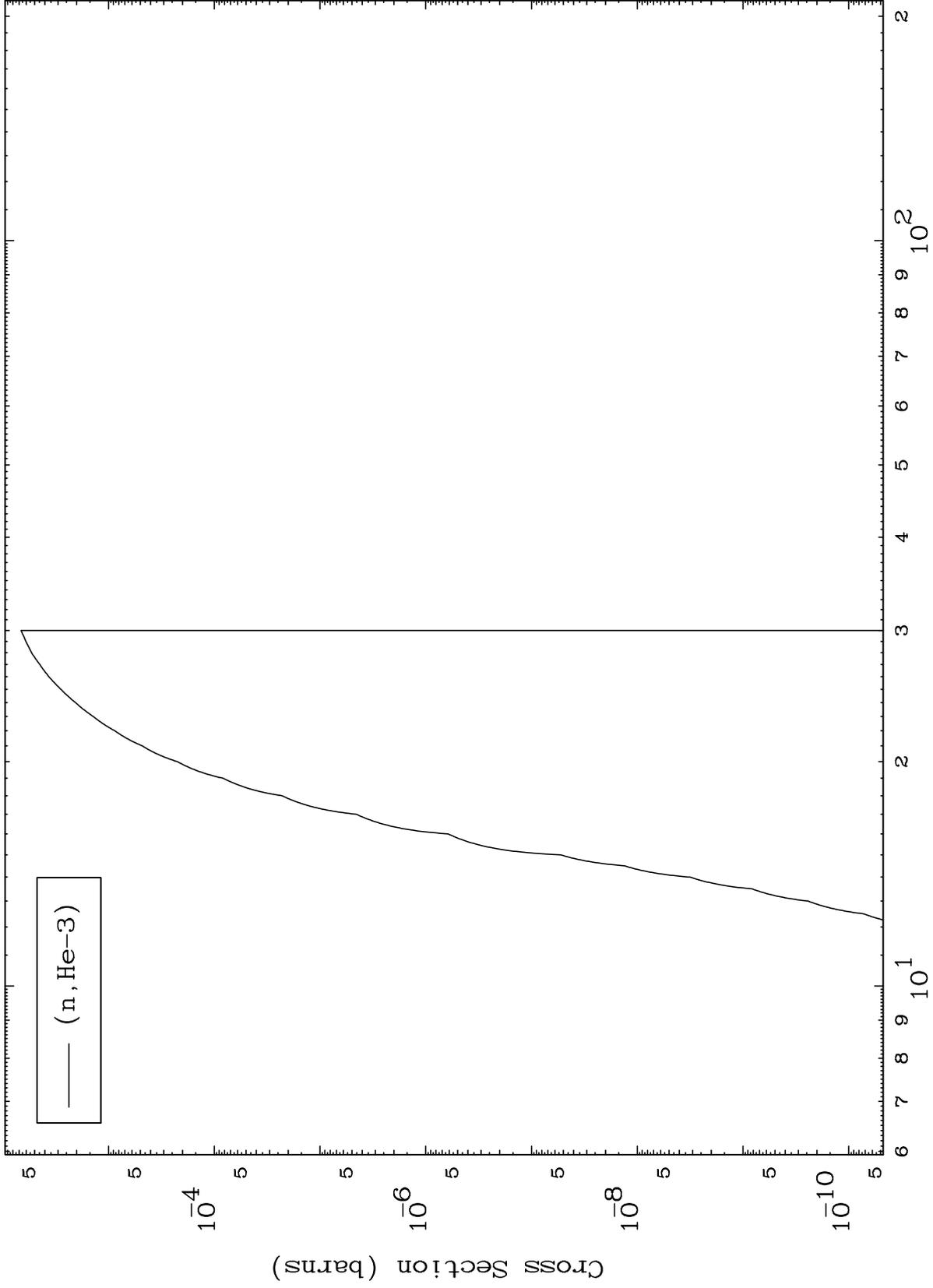
Incident Energy (MeV)

37-Rb-81m

MAT 3714

(n,He3) Levels
293 Kelvin Cross Sections

37-Rb-81m



15

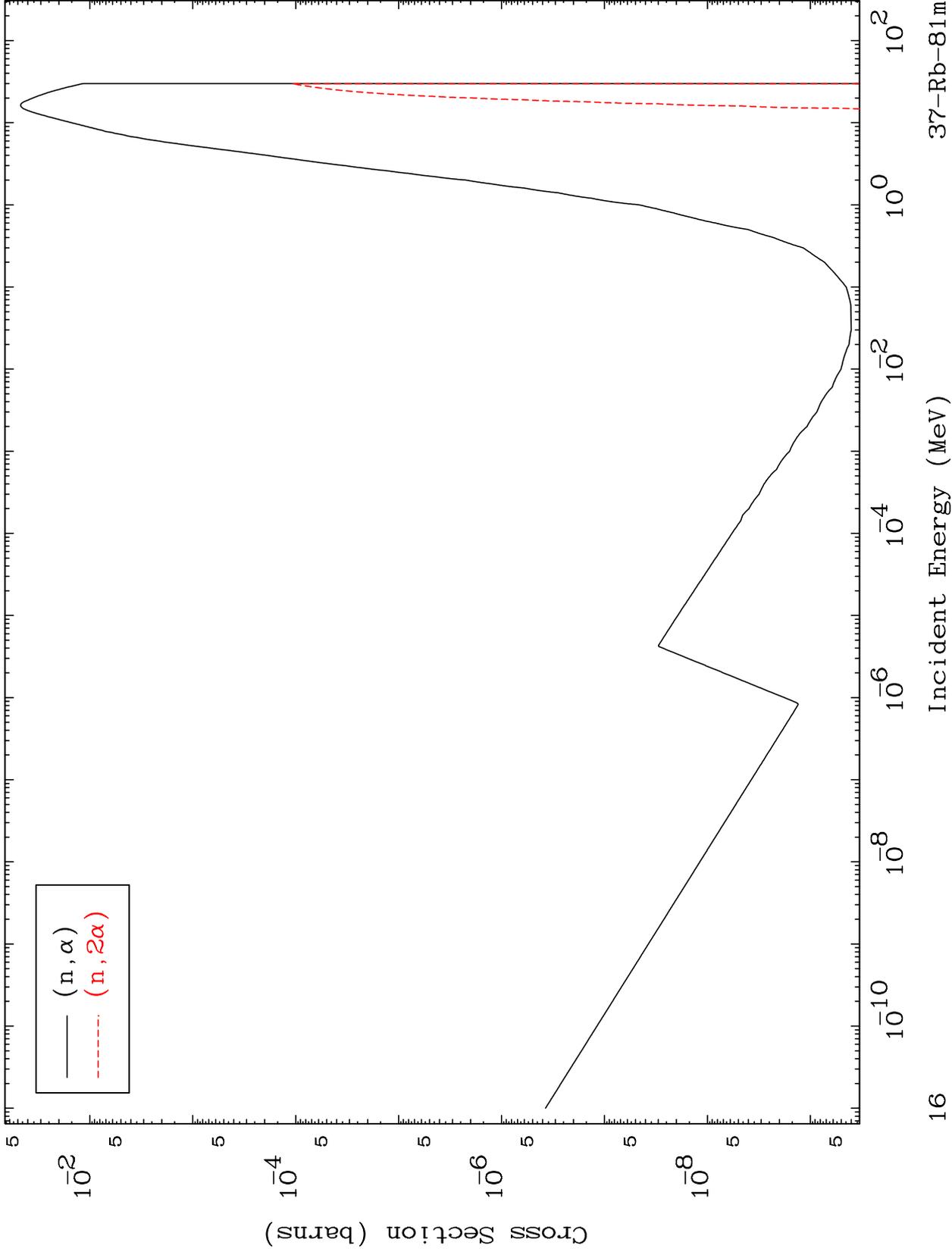
Incident Energy (MeV)

37-Rb-81m

MAT 3714

(n, α) Levels
293 Kelvin Cross Sections

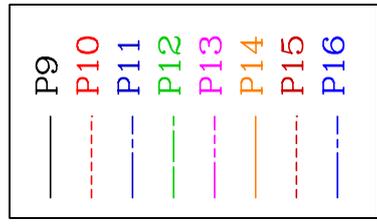
37-Rb-81m



MAT 3714

Elastic Legendre Coefficients

37-Rb-81m



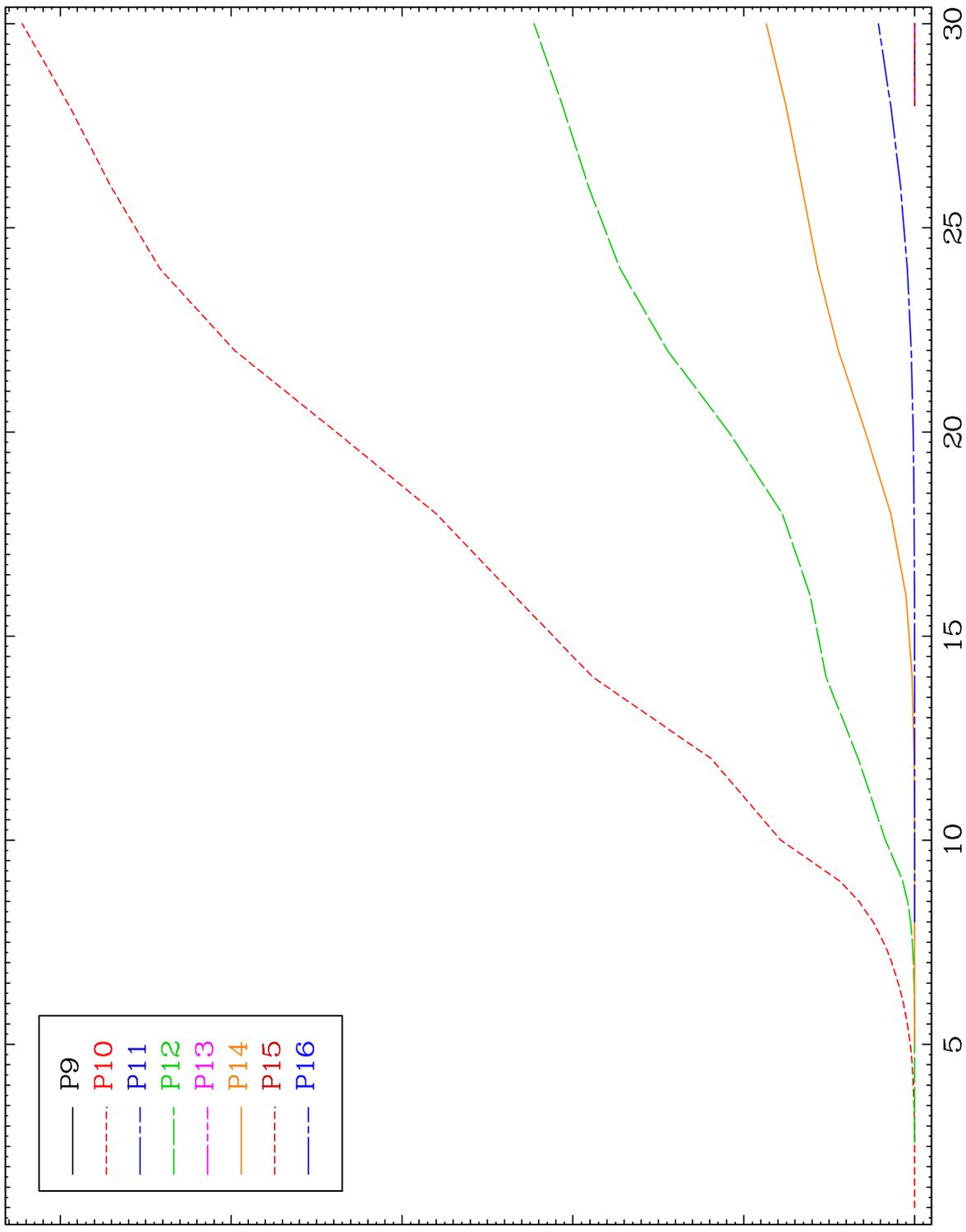
$\times 10^{-45}$

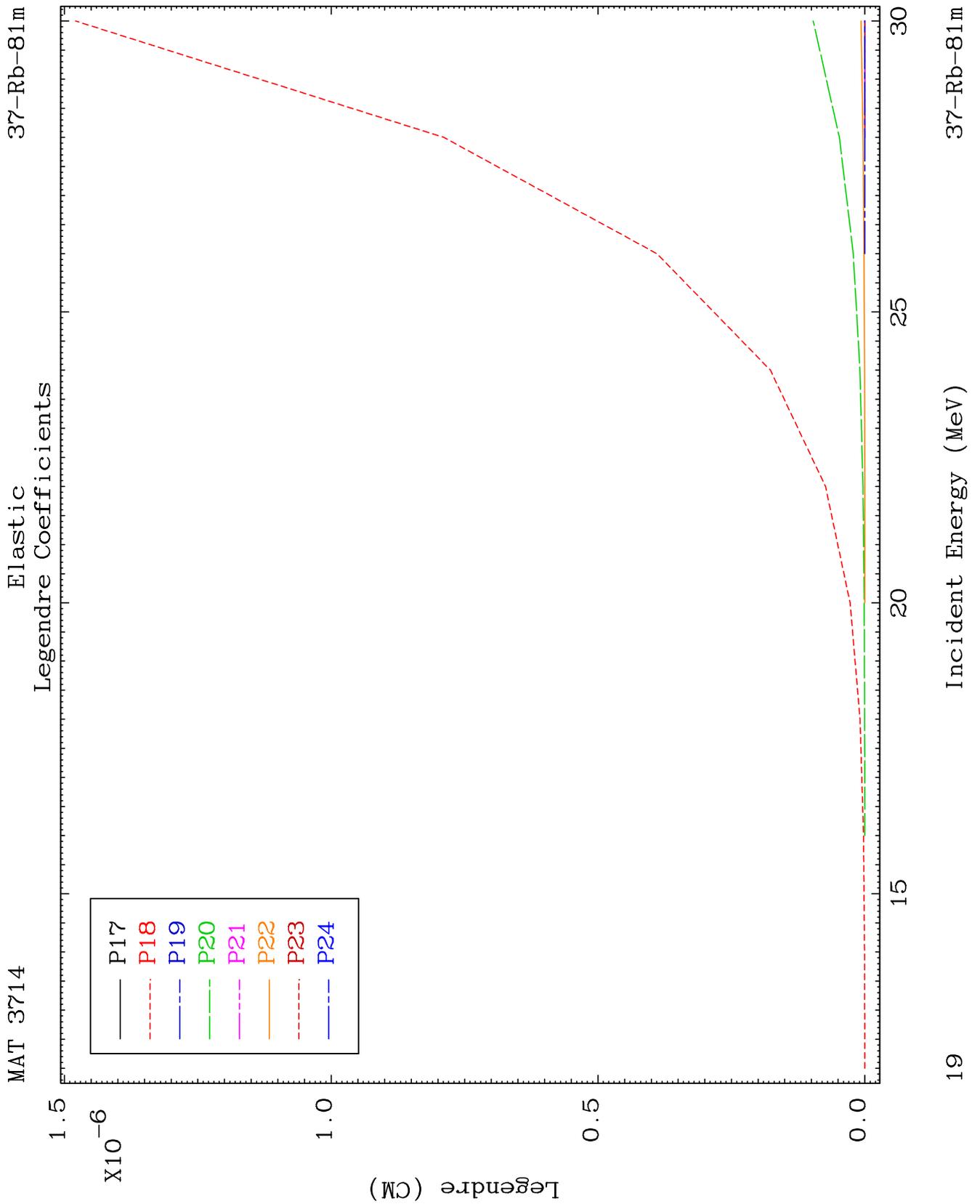
Legendre (CM)

18

Incident Energy (MeV)

37-Rb-81m

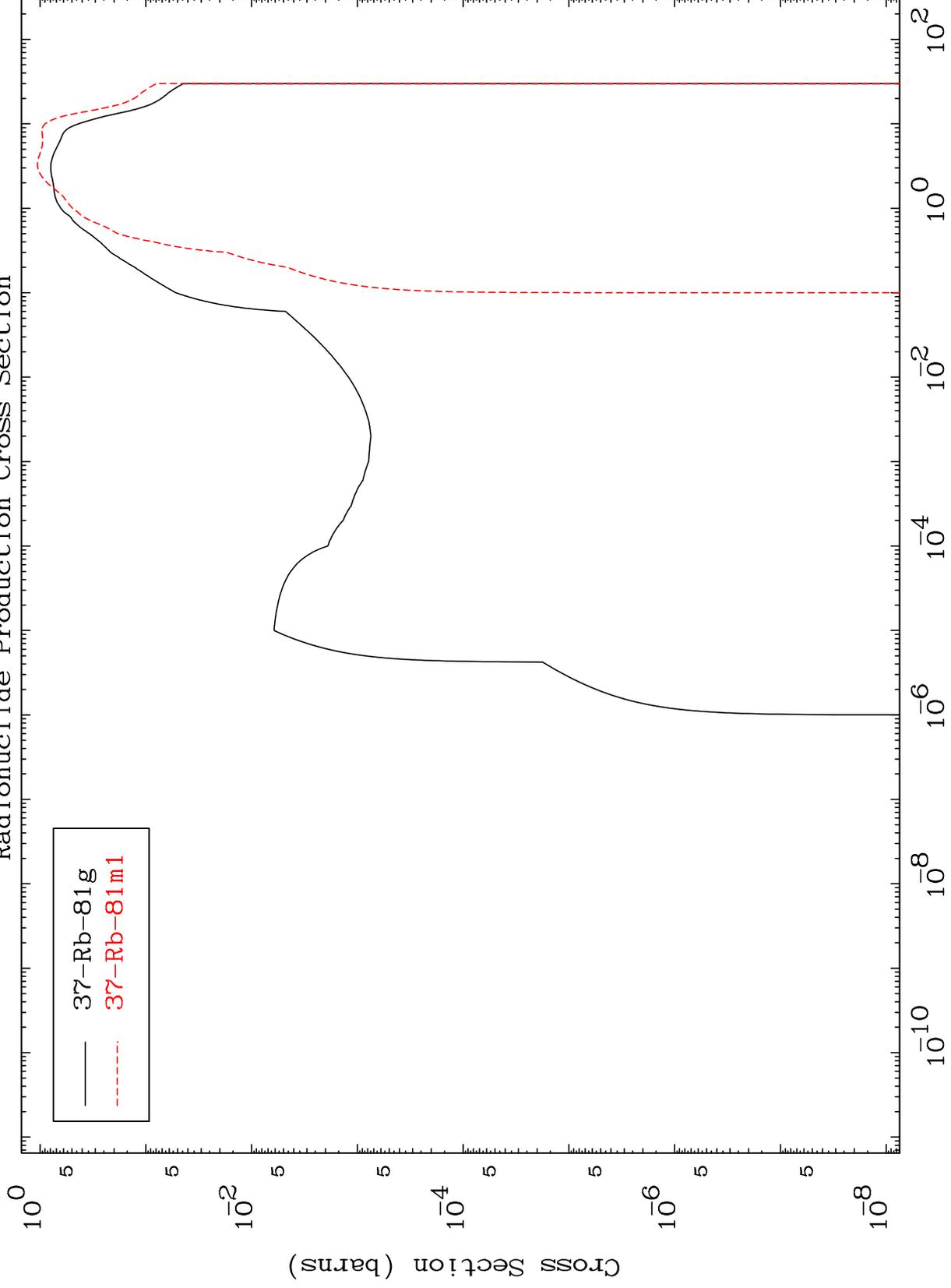




MAT 3714

Inelastic
Radionuclide Production Cross Section

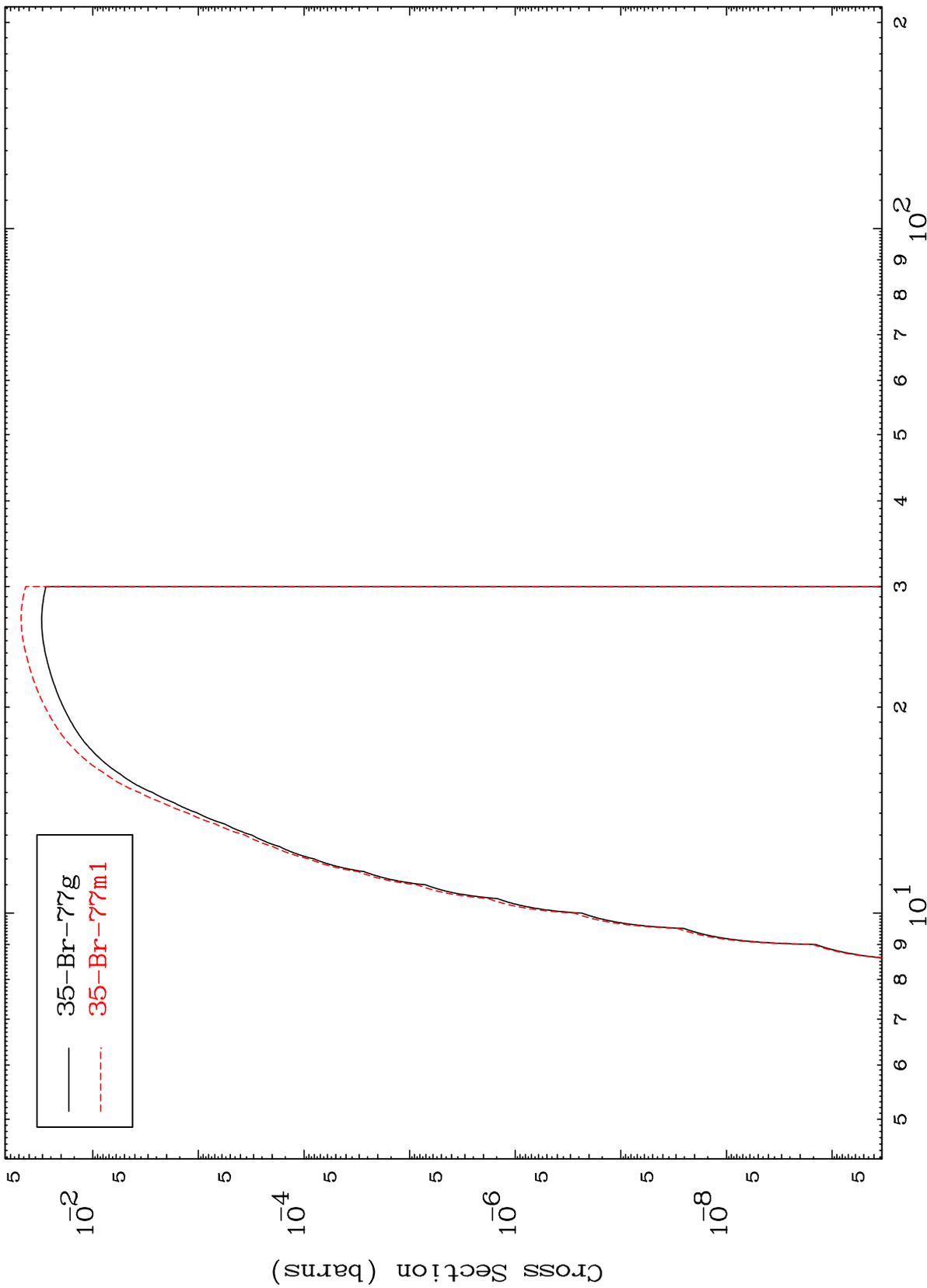
³⁷Rb-81m



MAT 3714

37-Rb-81m

$(n, n') \alpha$
Radionuclide Production Cross Section



37-Rb-81m

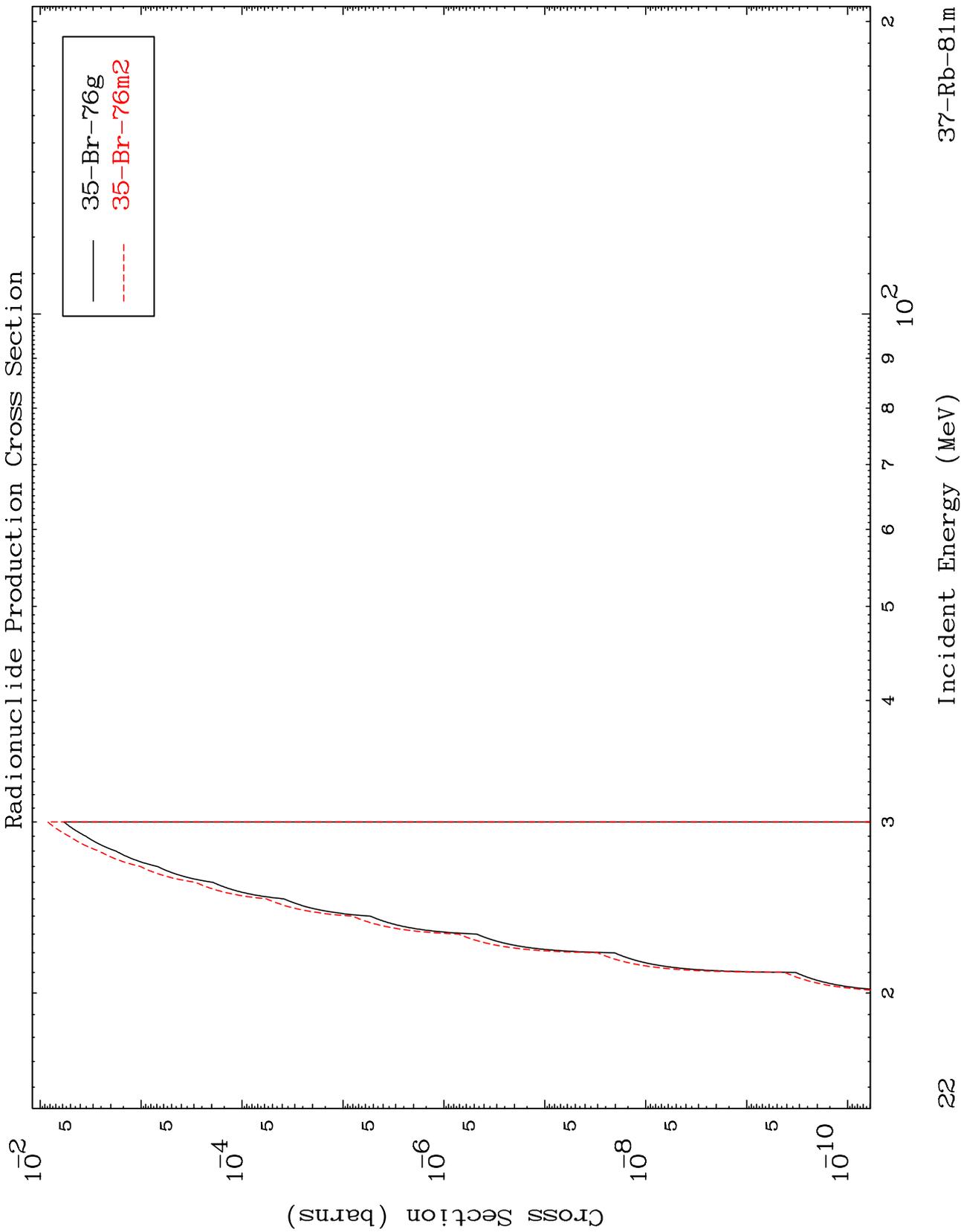
Incident Energy (MeV)

21

MAT 3714

$(n,2n) \alpha$

37-Rb-81m



22

Incident Energy (MeV)

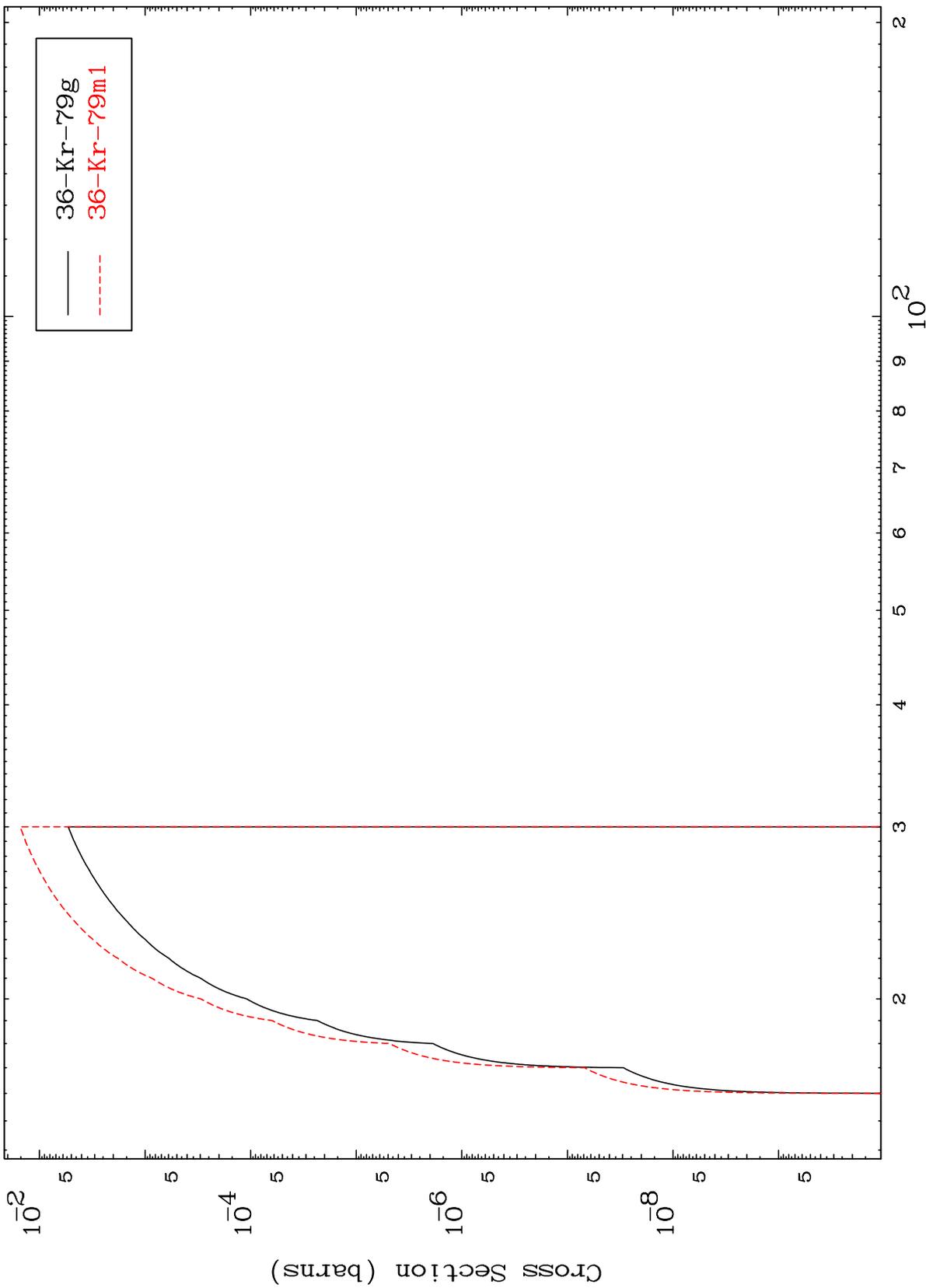
37-Rb-81m

MAT 3714

(n,n') d

37-Rb-81m

Radionuclide Production Cross Section



23

Incident Energy (MeV)

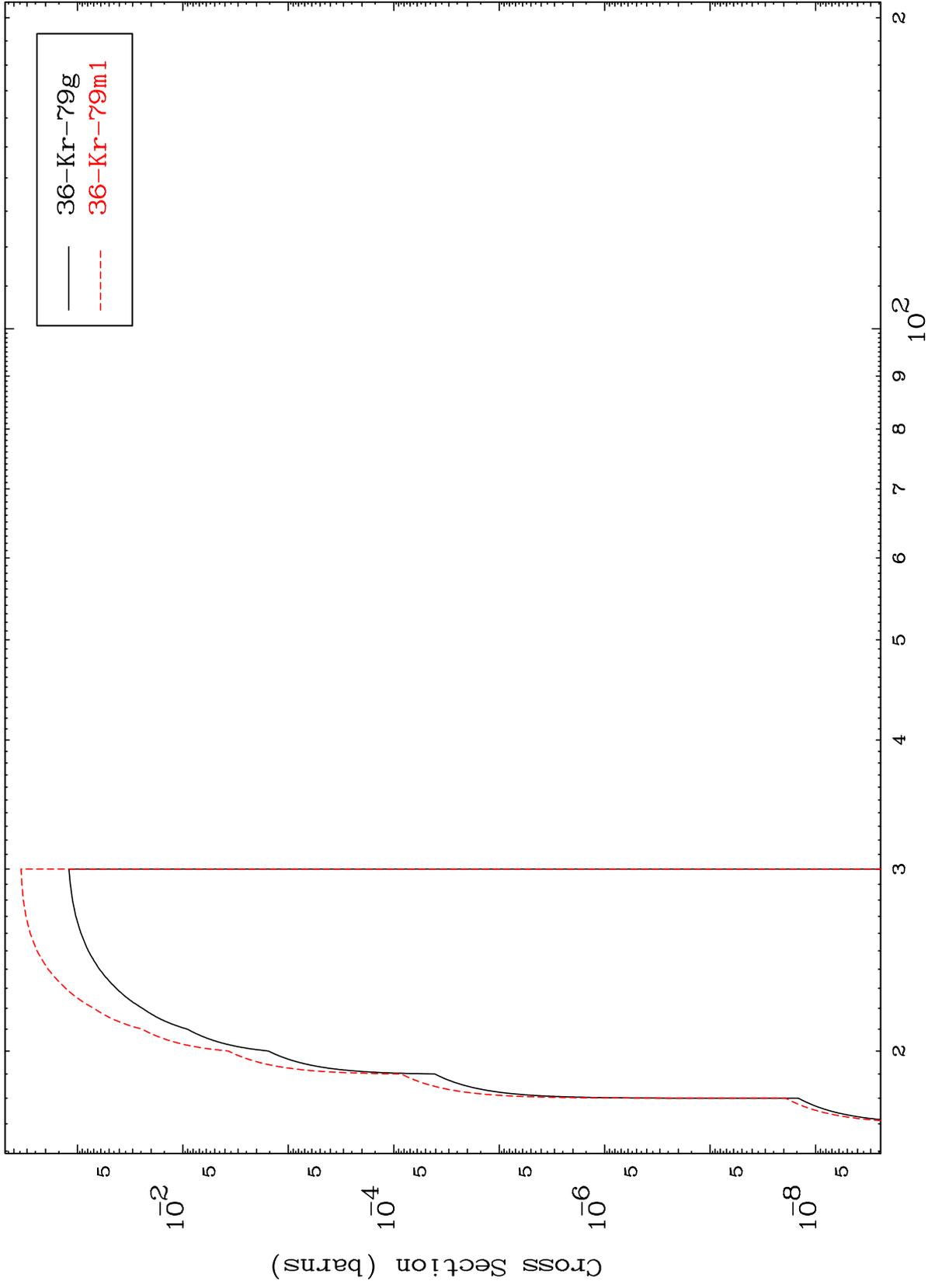
37-Rb-81m

MAT 3714

(n,2n) p

37-Rb-81m

Radionuclide Production Cross Section



24

Incident Energy (MeV)

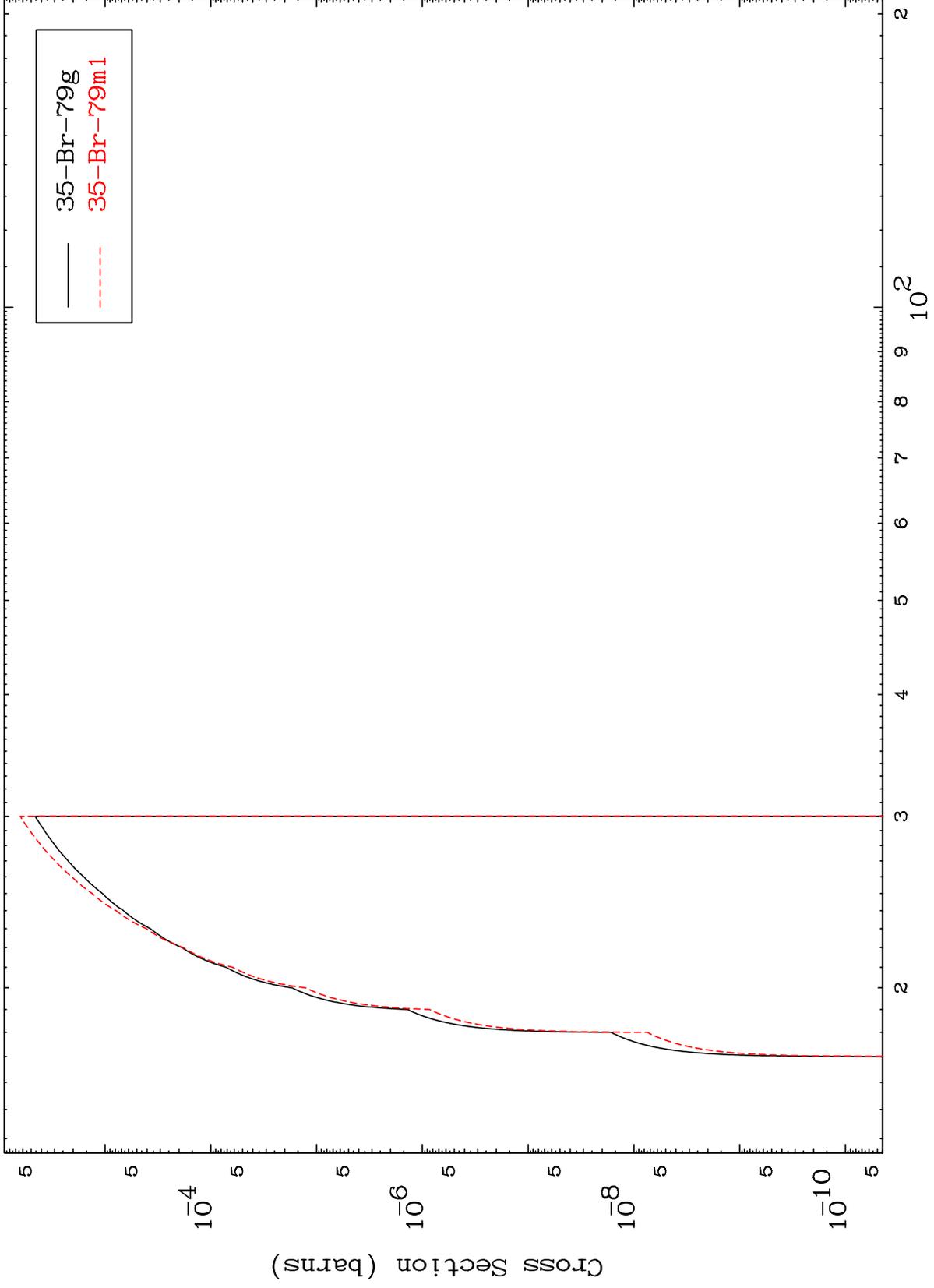
37-Rb-81m

MAT 3714

(n,2n) p

37-Rb-81m

Radionuclide Production Cross Section



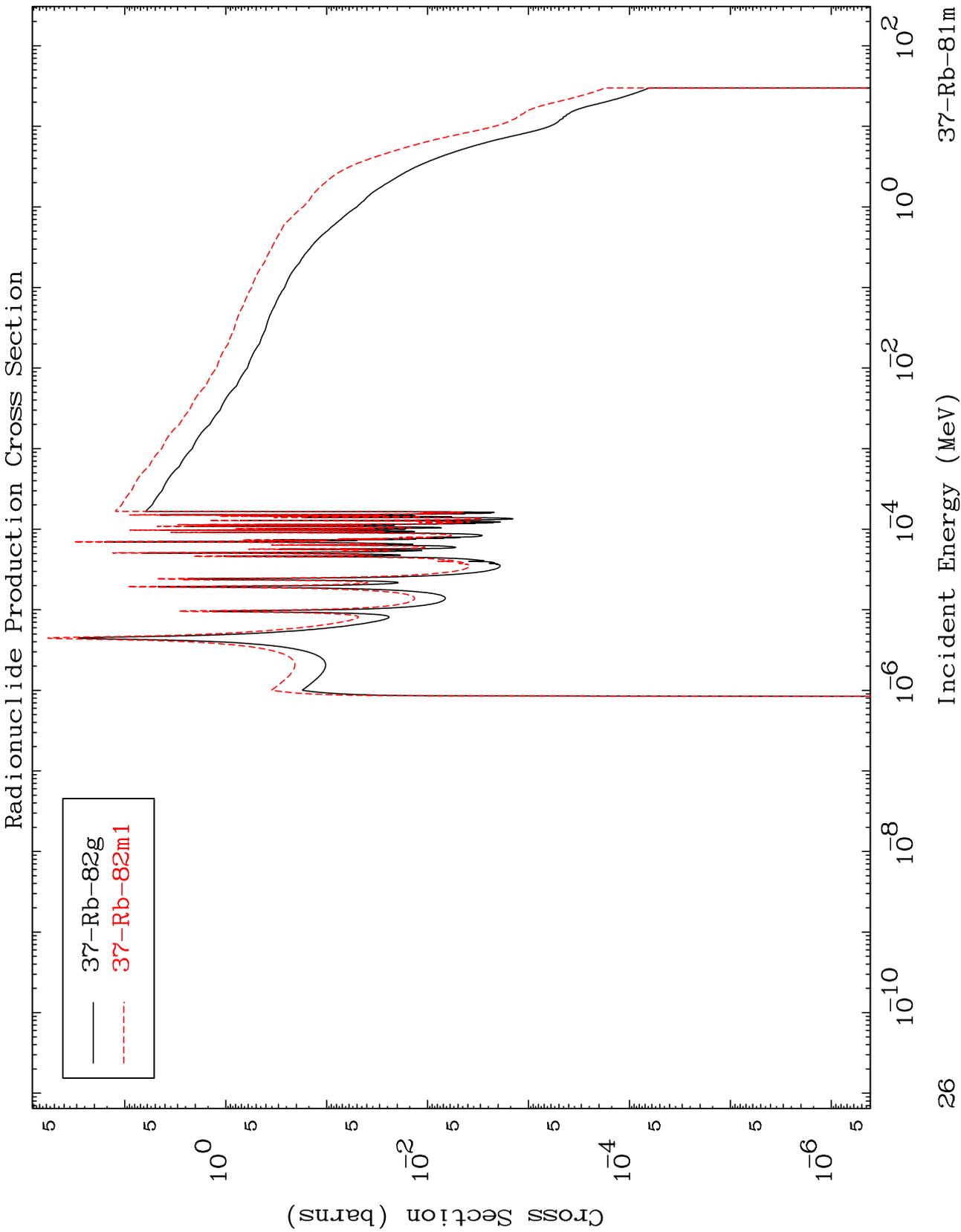
25

Incident Energy (MeV)

37-Rb-81m

MAT 3714

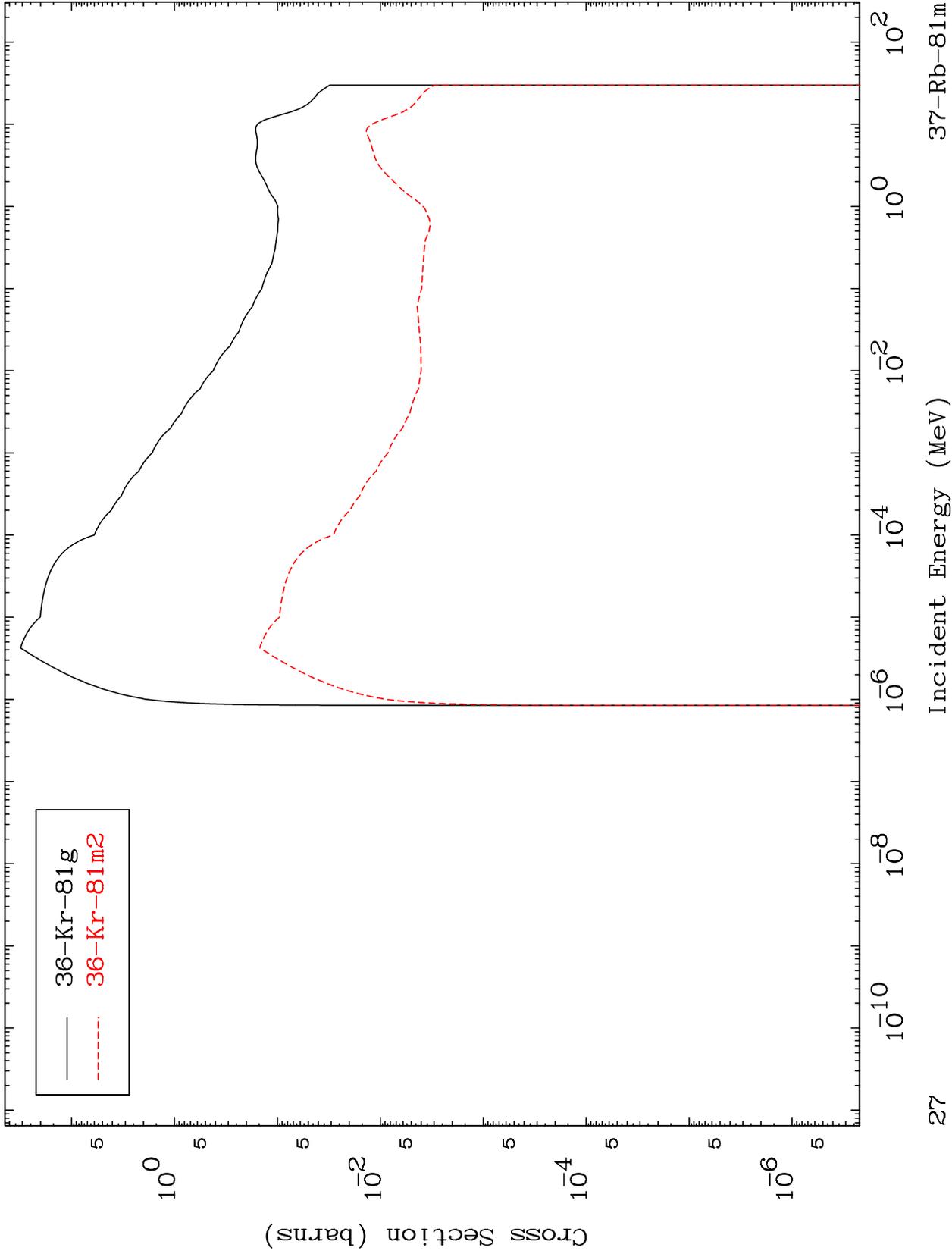
³⁷Rb-81m



MAT 3714

37-Rb-81m

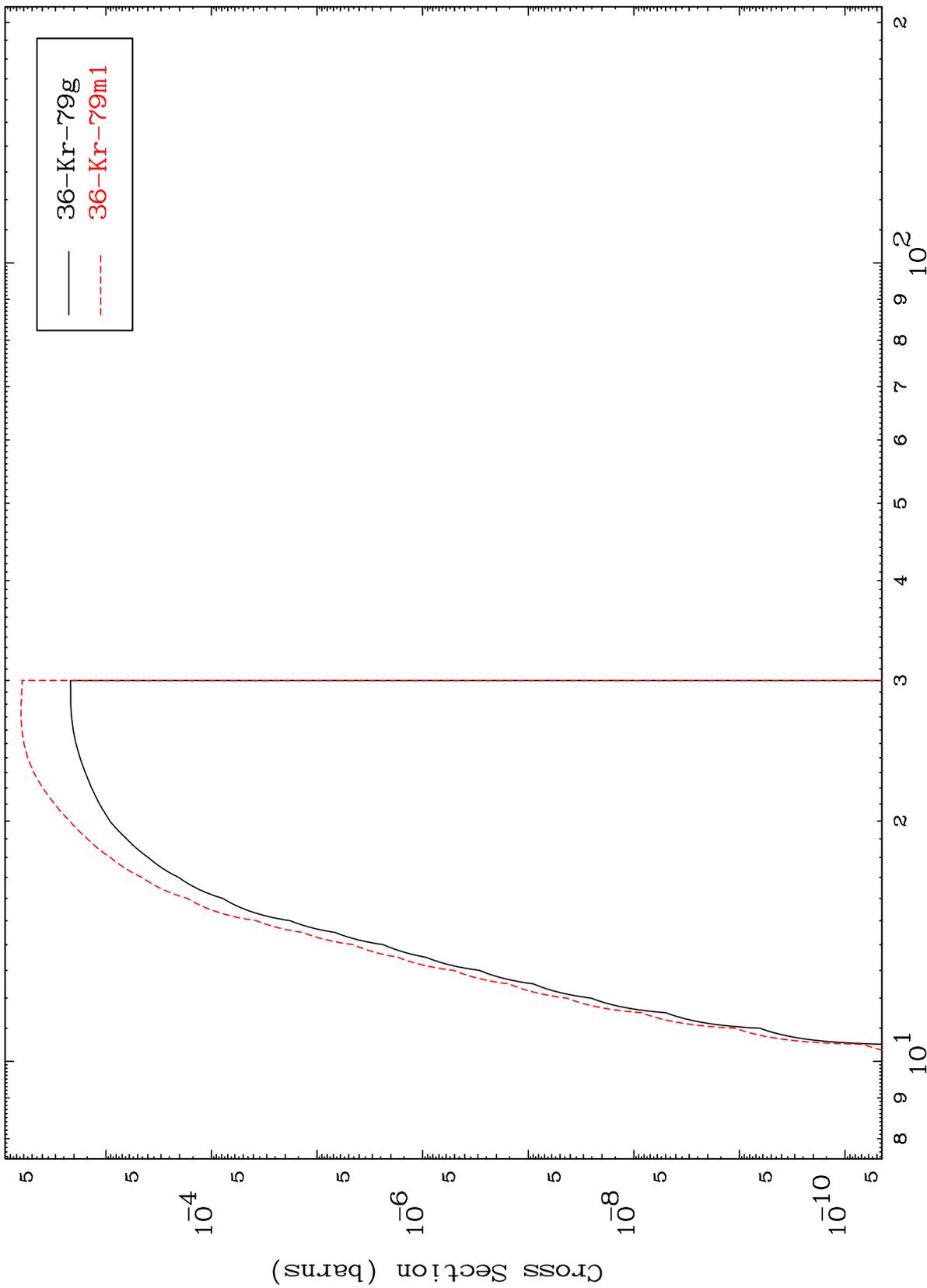
(n,p)
Radionuclide Production Cross Section



MAT 3714

37-Rb-81m

(n,t)
Radionuclide Production Cross Section



28

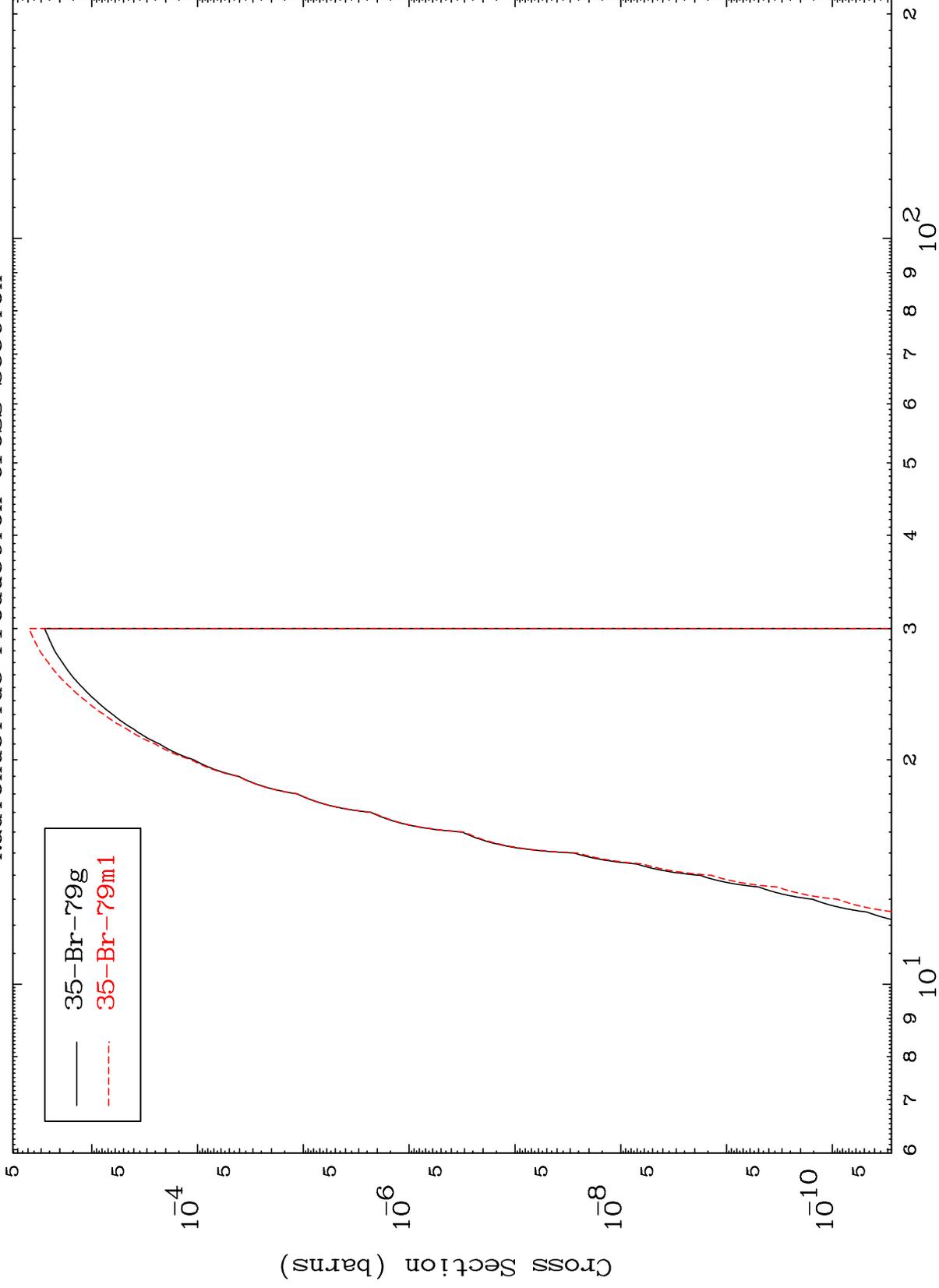
Incident Energy (MeV)

37-Rb-81m

MAT 3714

37-Rb-81m

Radionuclide Production Cross Section
(n,He-3)



29

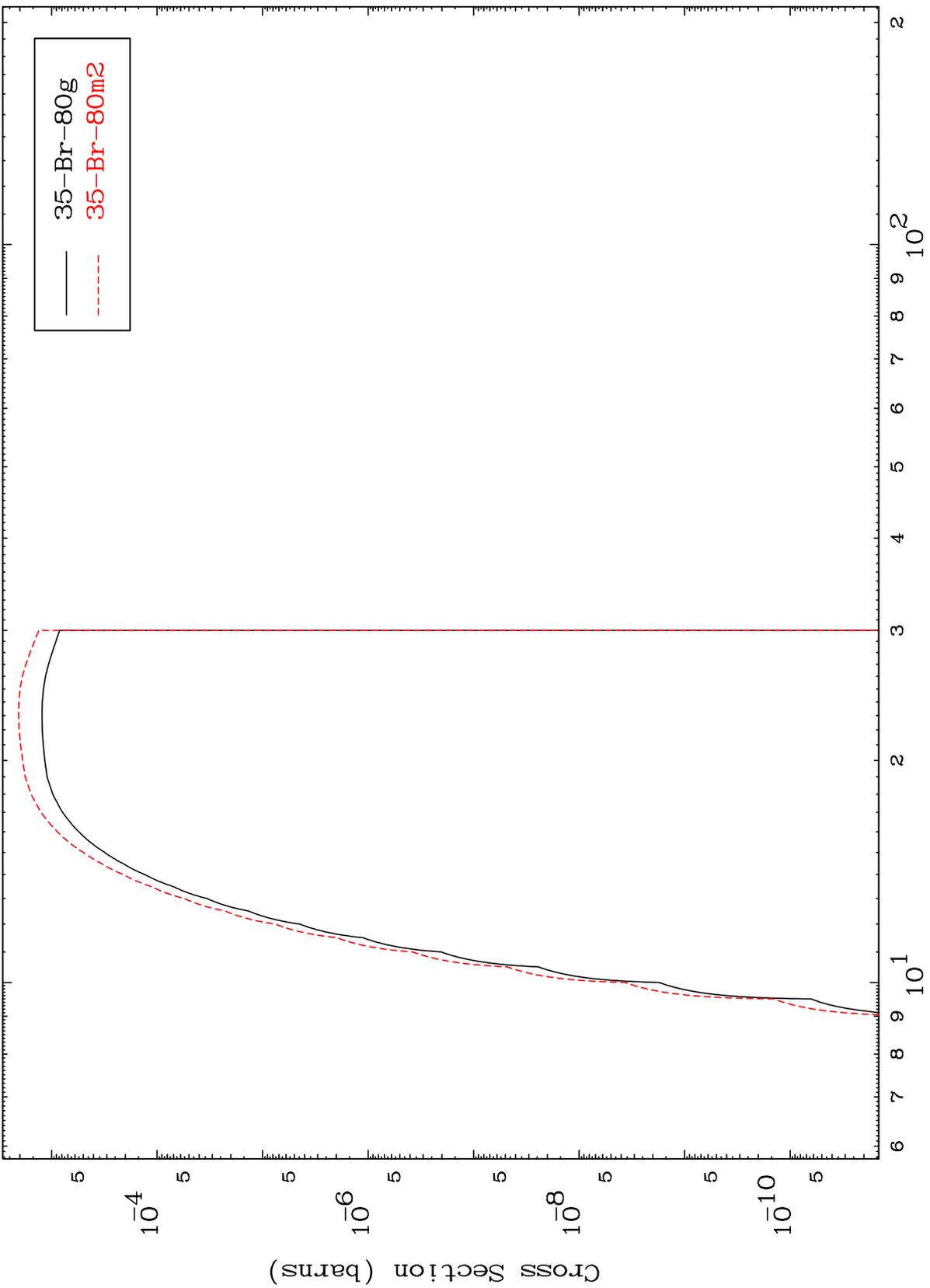
Incident Energy (MeV)

37-Rb-81m

MAT 3714

37-Rb-81m

(n,2p)
Radionuclide Production Cross Section



30

Incident Energy (MeV)

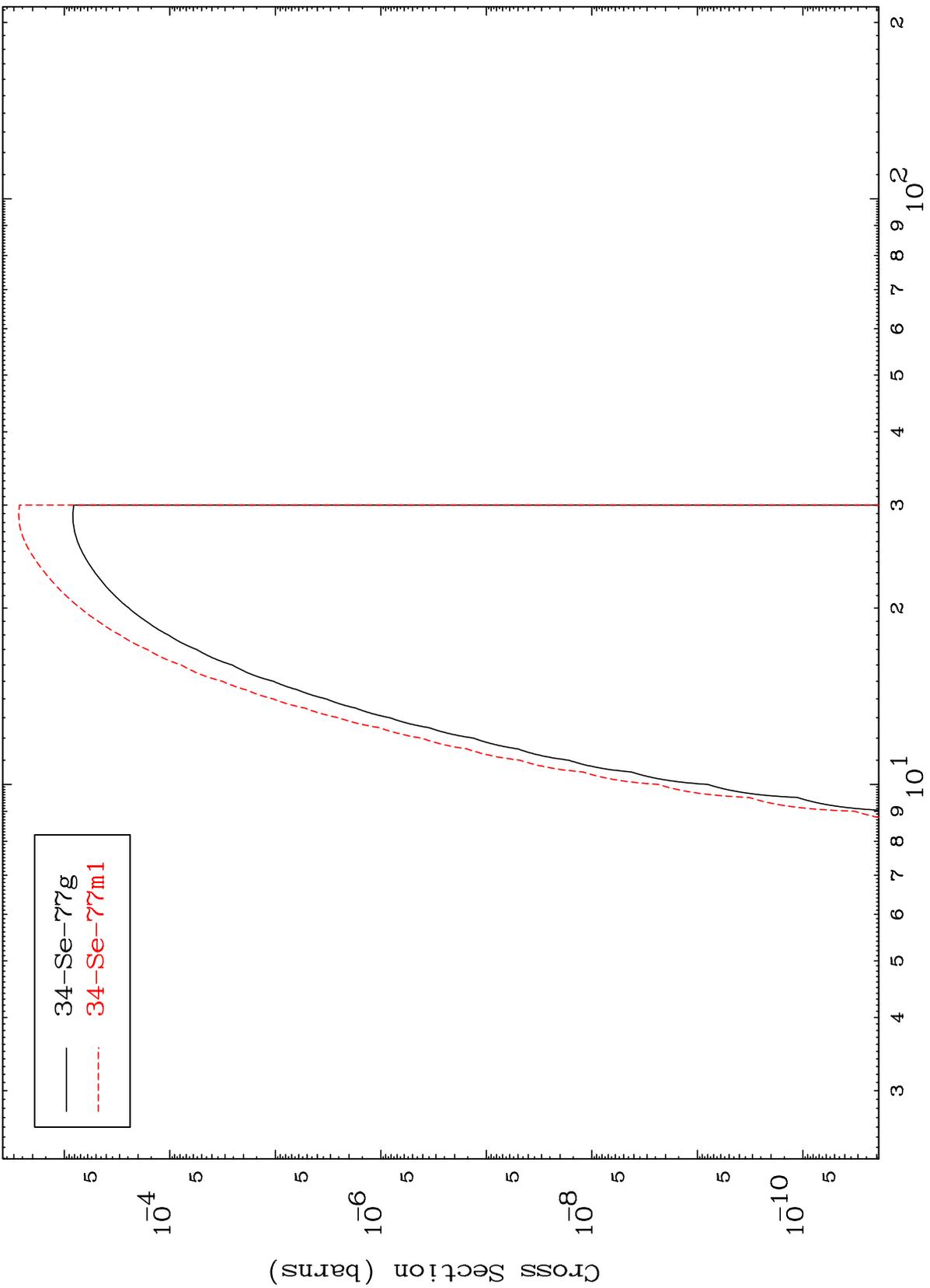
37-Rb-81m

MAT 3714

(n,p) α

37-Rb-81m

Radionuclide Production Cross Section



31

Incident Energy (MeV)

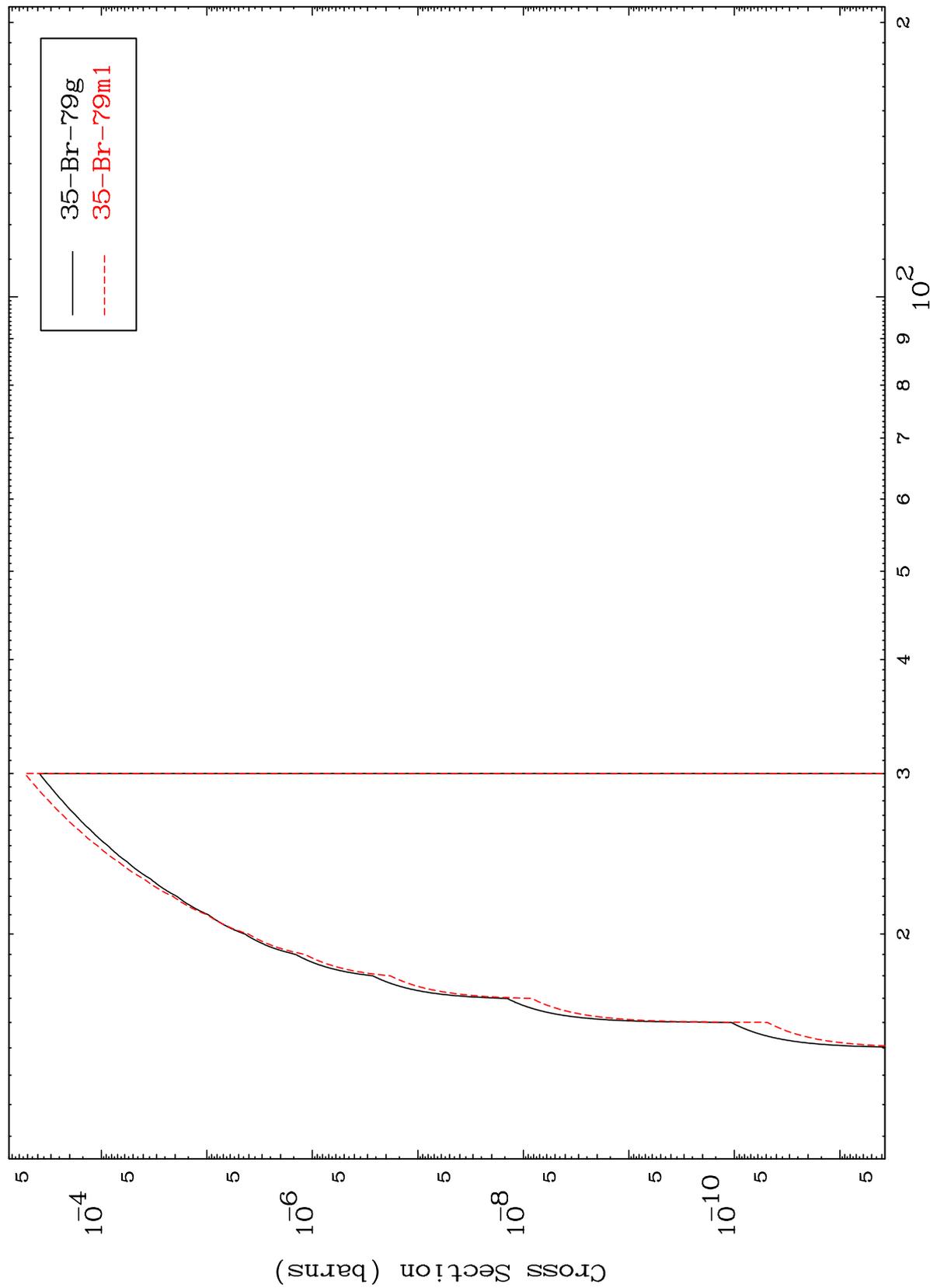
37-Rb-81m

MAT 3714

(n,p) d

37-Rb-81m

Radionuclide Production Cross Section



32

Incident Energy (MeV)

37-Rb-81m