

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

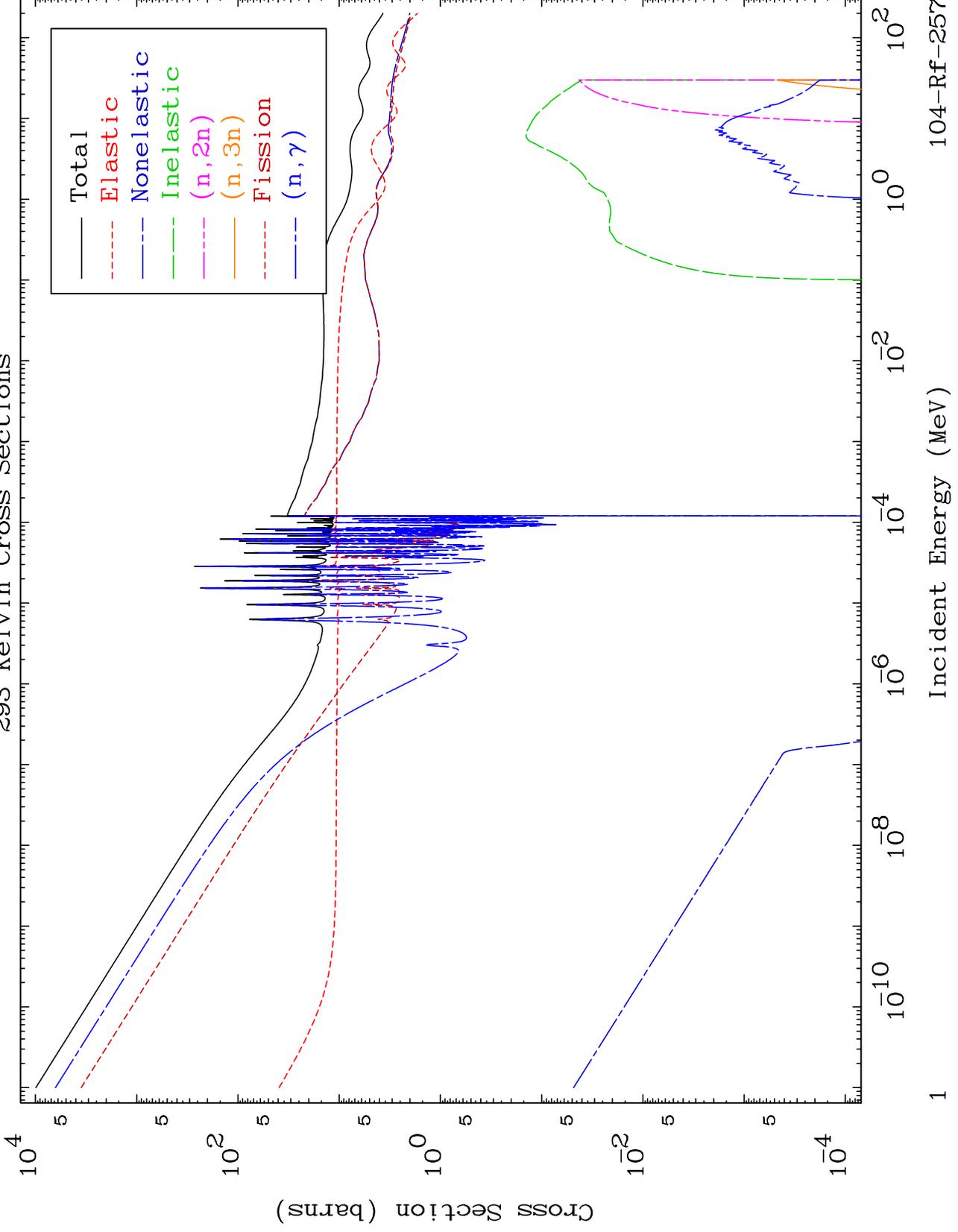
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Press Mouse Button to Start

MAT 457

Neutron Major
293 Kelvin Cross Sections

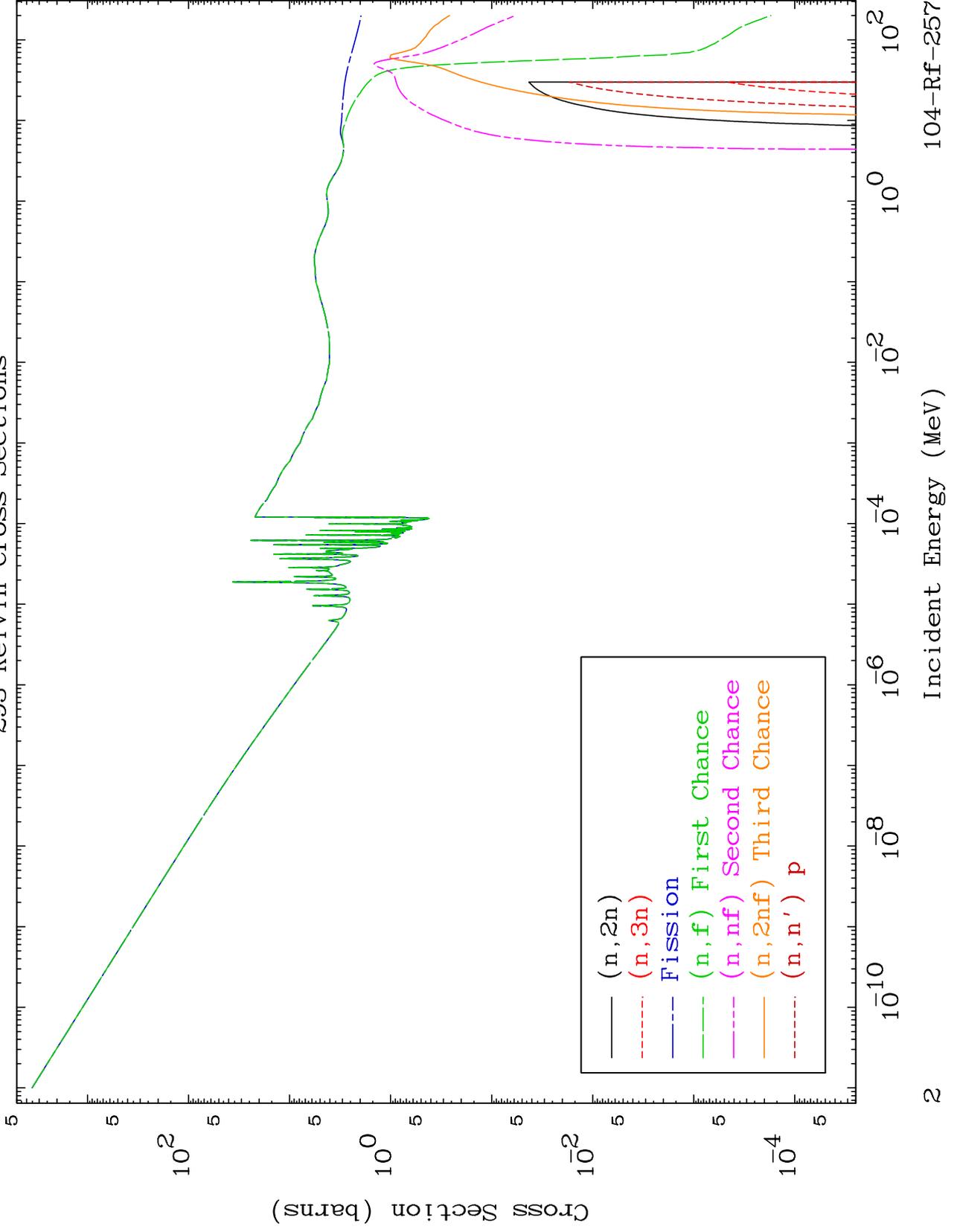
104-Rf-257

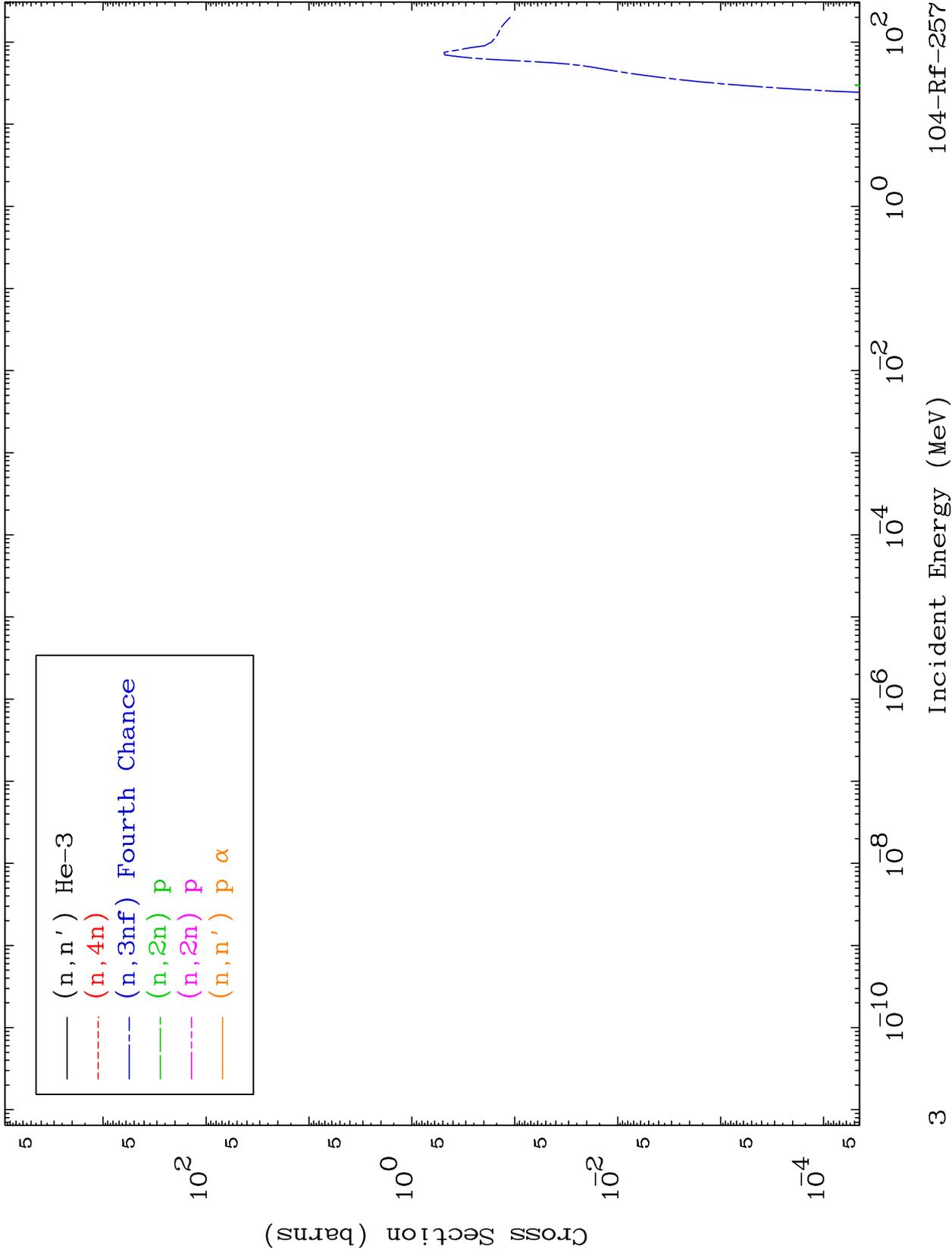


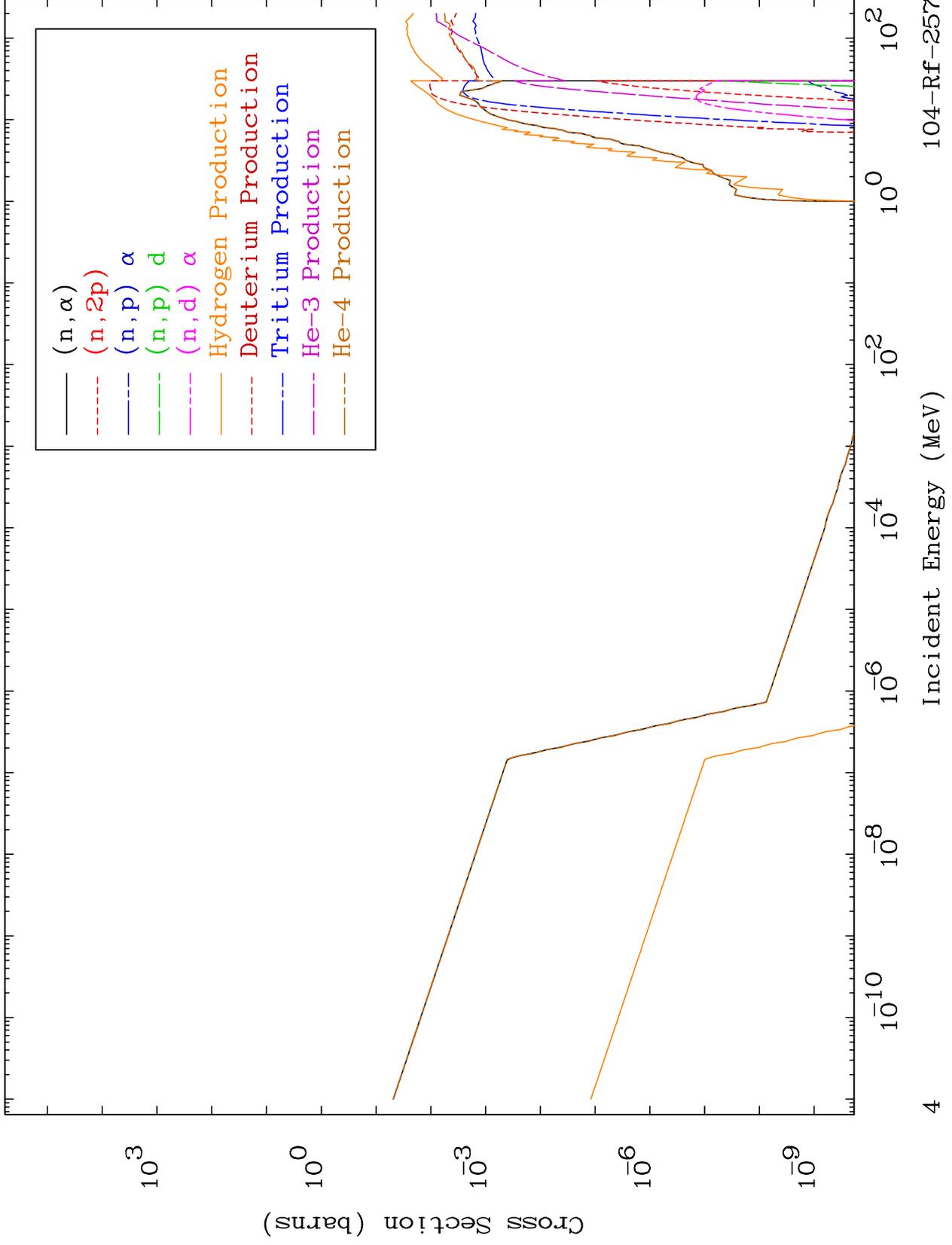
MAT 457

Neutron Absorption
293 Kelvin Cross Sections

104-Rf-257



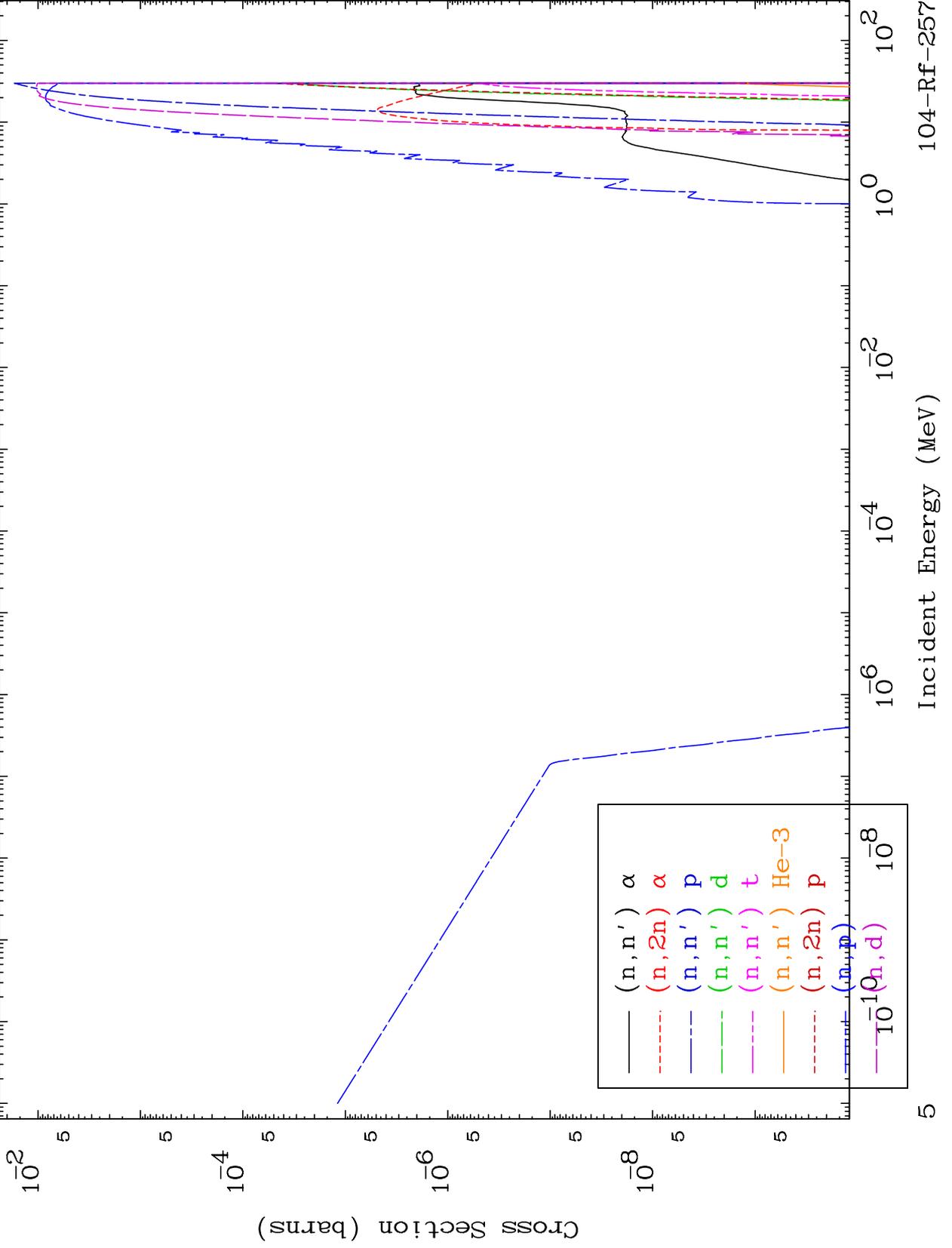




MAT 457

Charged Particle
293 Kelvin Cross Sections

104-Rf-257

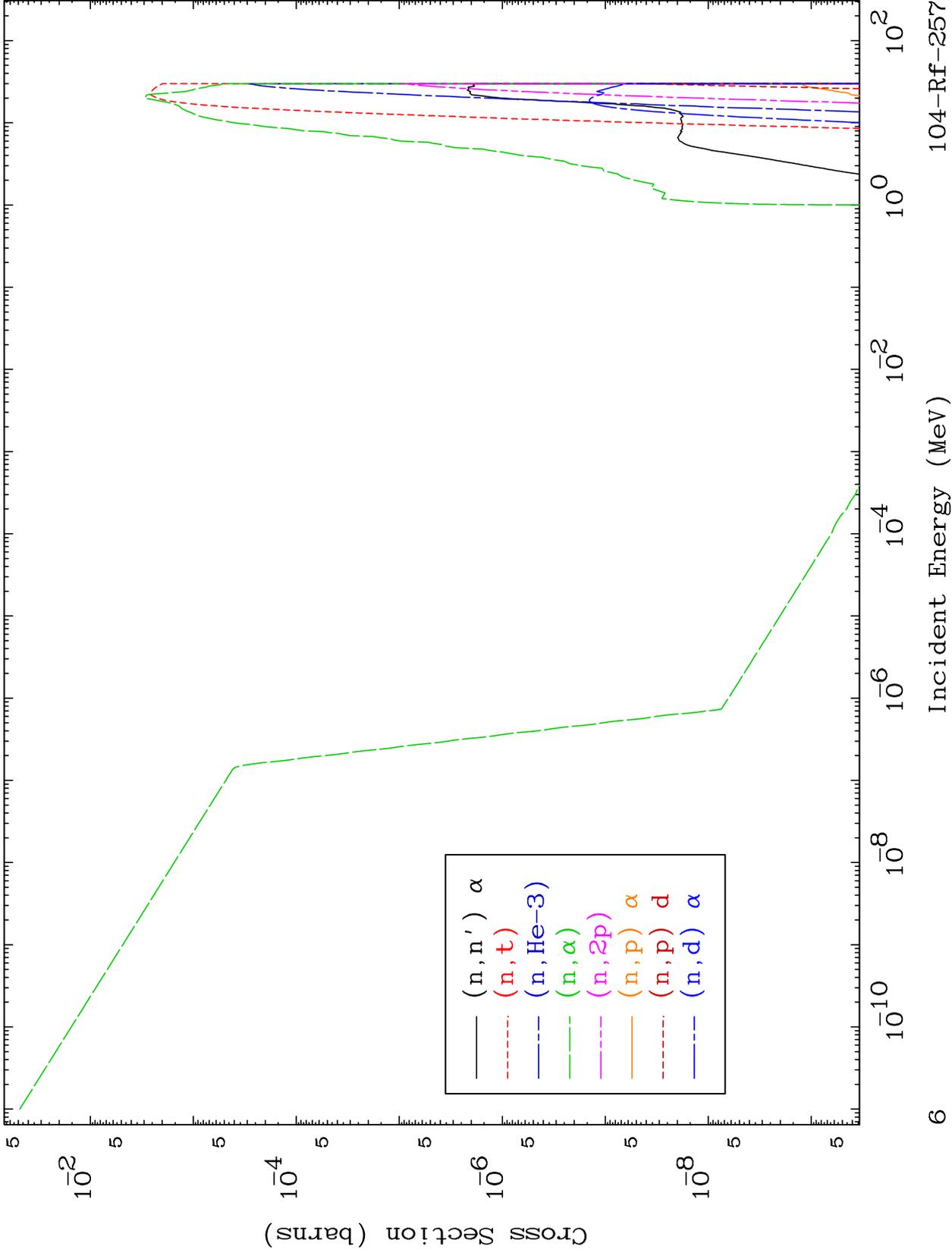


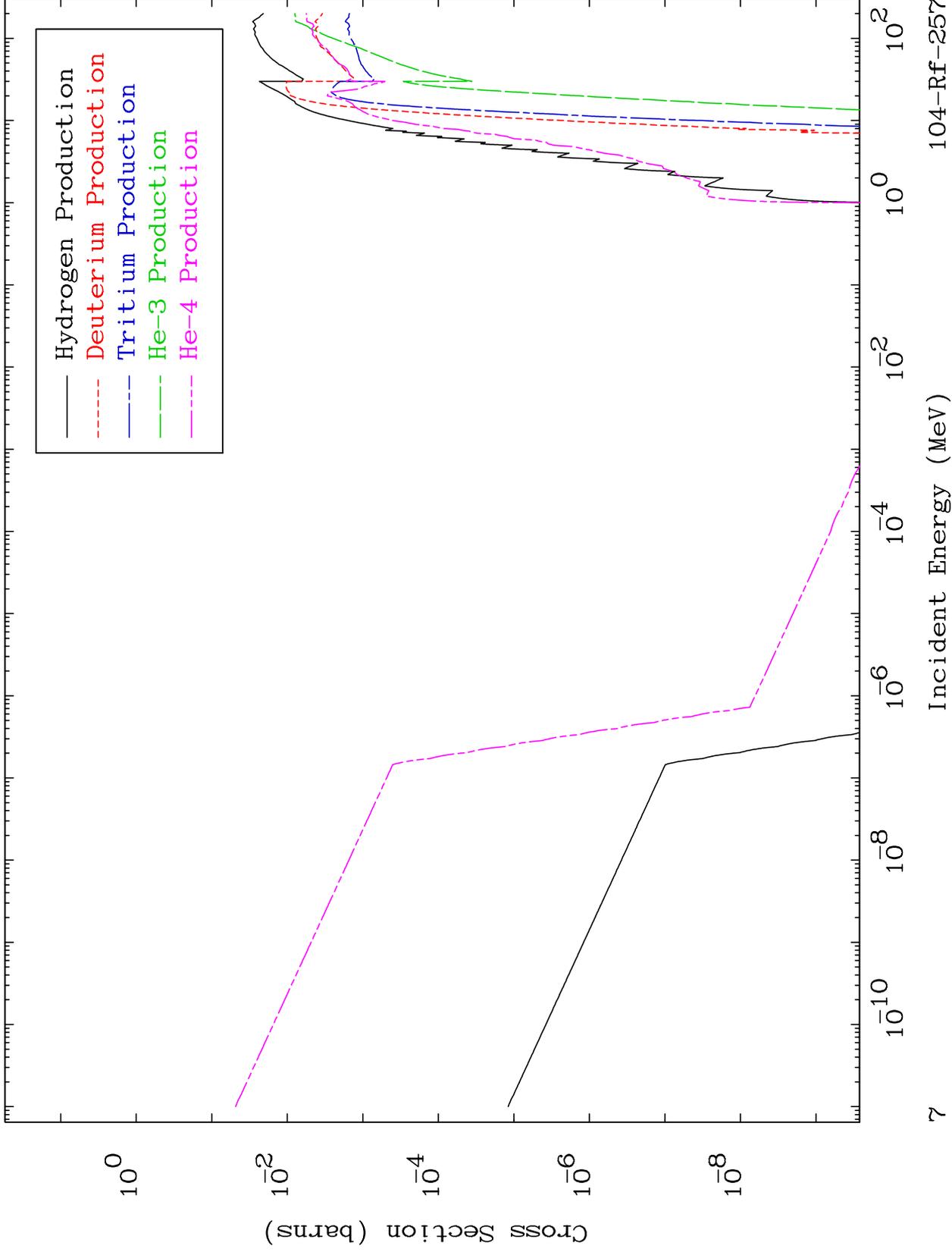
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MAT 457

Charged Particle
293 Kelvin Cross Sections

104-Rf-257

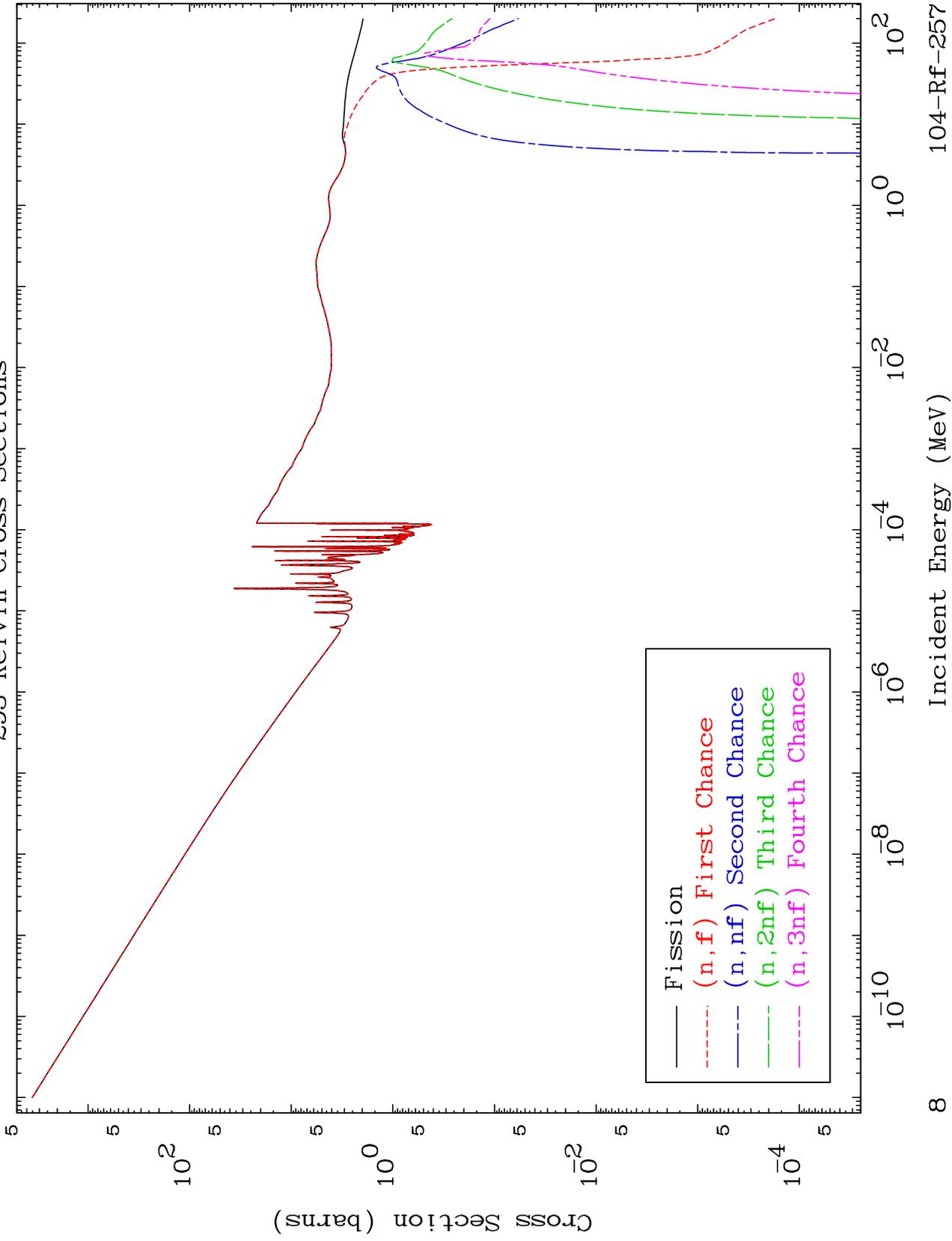




MAT 457

Fission
293 Kelvin Cross Sections

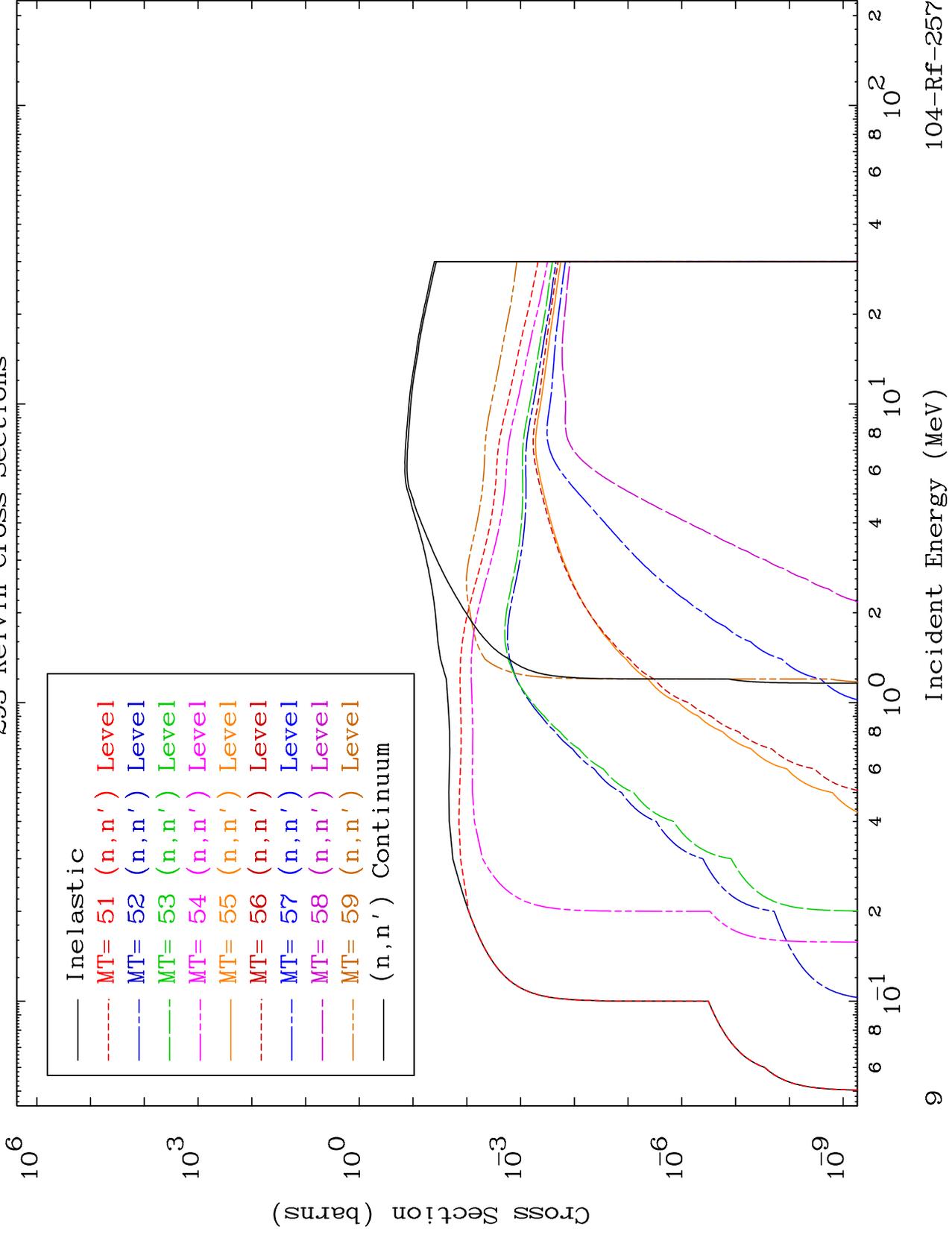
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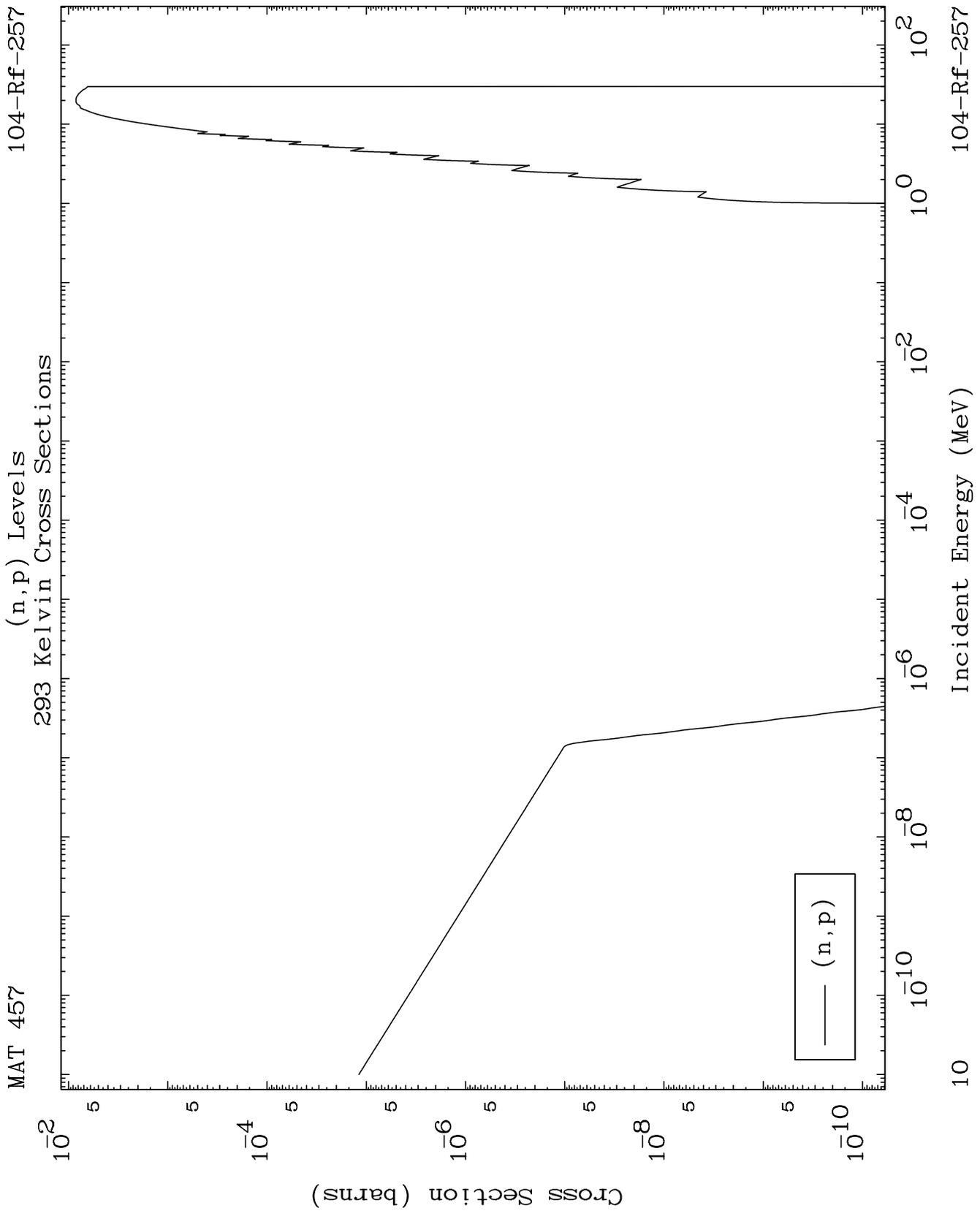


MAT 457

(n,n') Levels
293 Kelvin Cross Sections

104-Rf-257

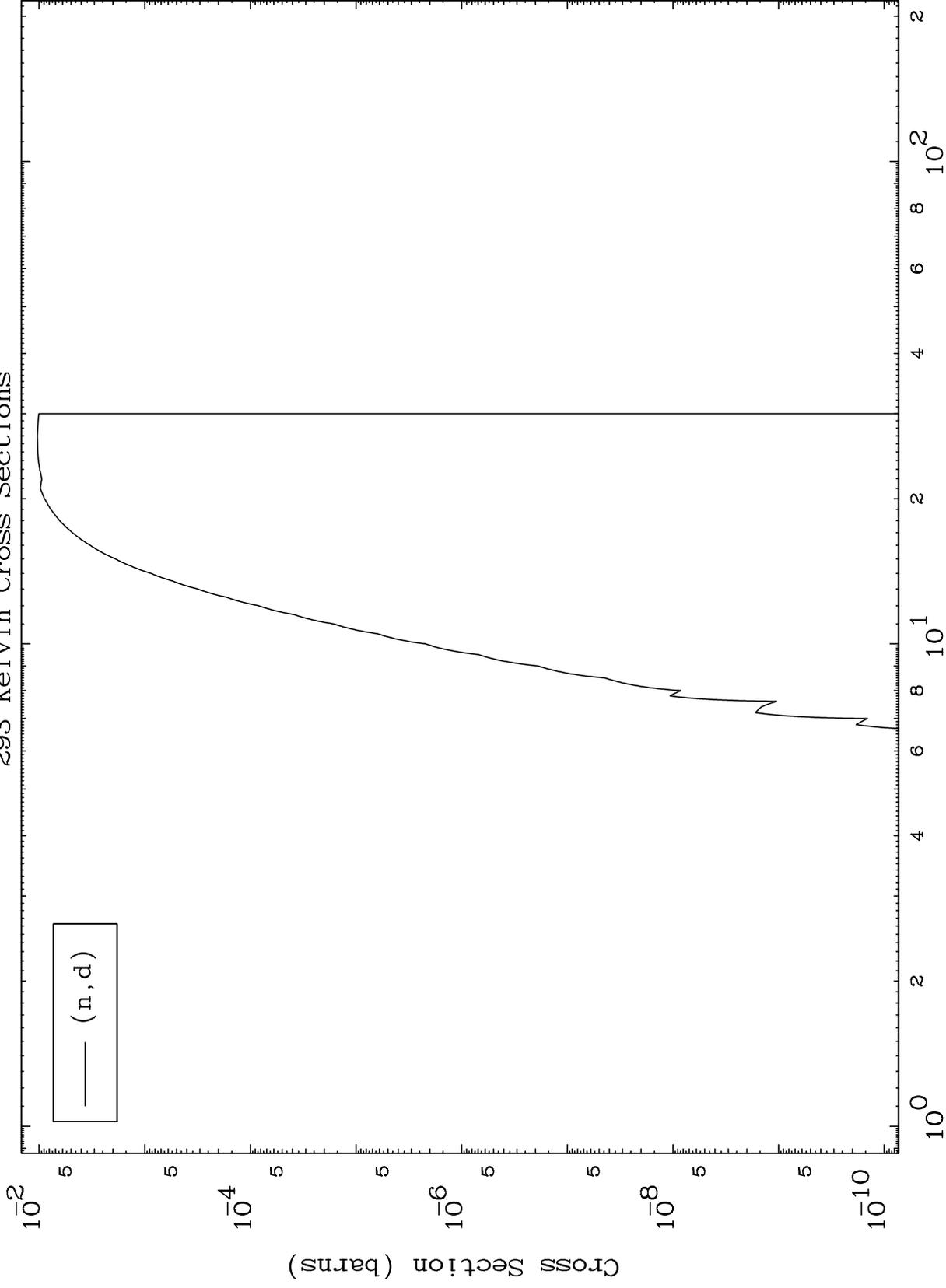




MAT 457

(n,d) Levels
293 Kelvin Cross Sections

104-Rf-257



11

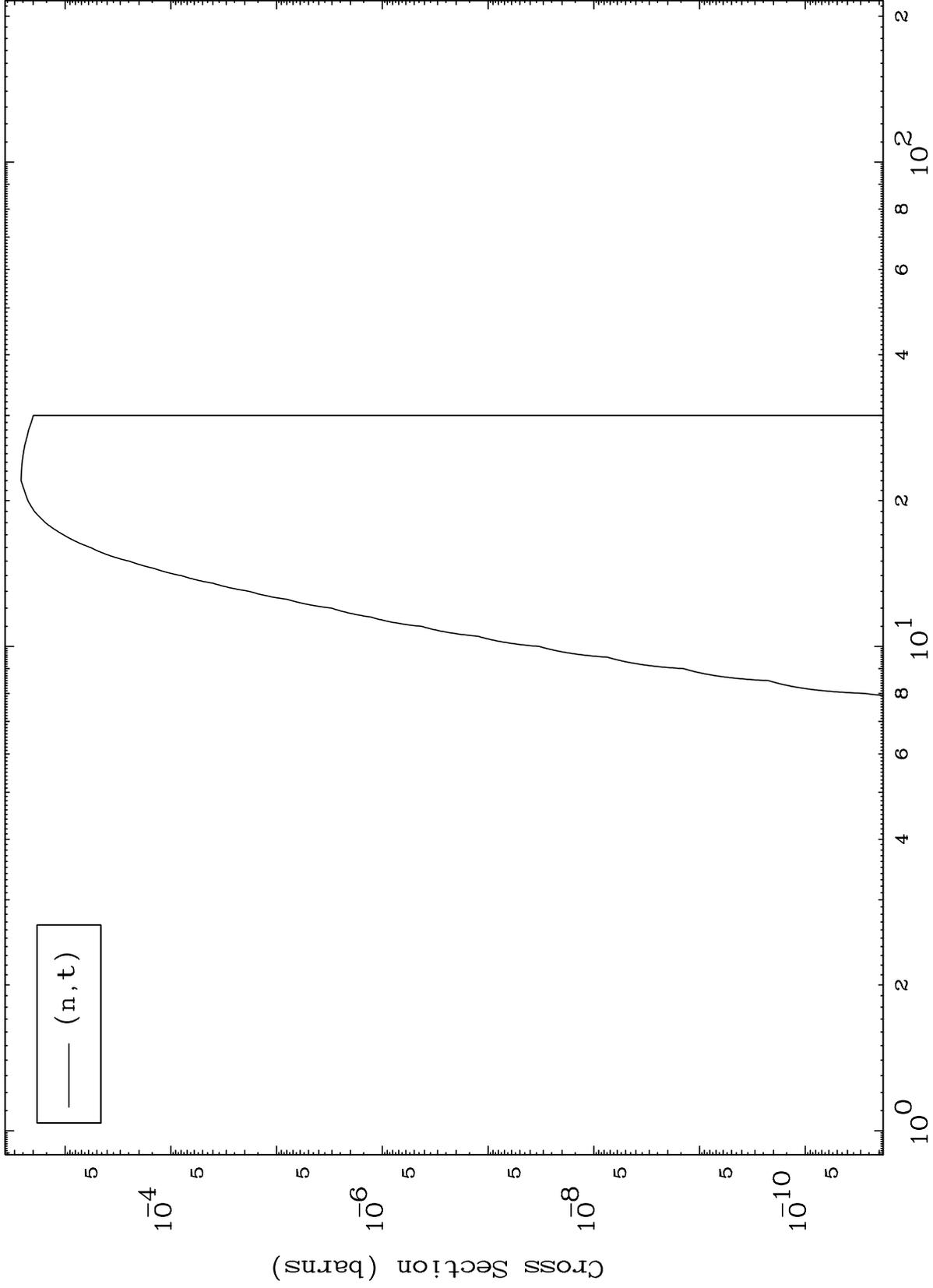
Incident Energy (MeV)

104-Rf-257

MAT 457

(n,t) Levels
293 Kelvin Cross Sections

104-Rf-257



Incident Energy (MeV)

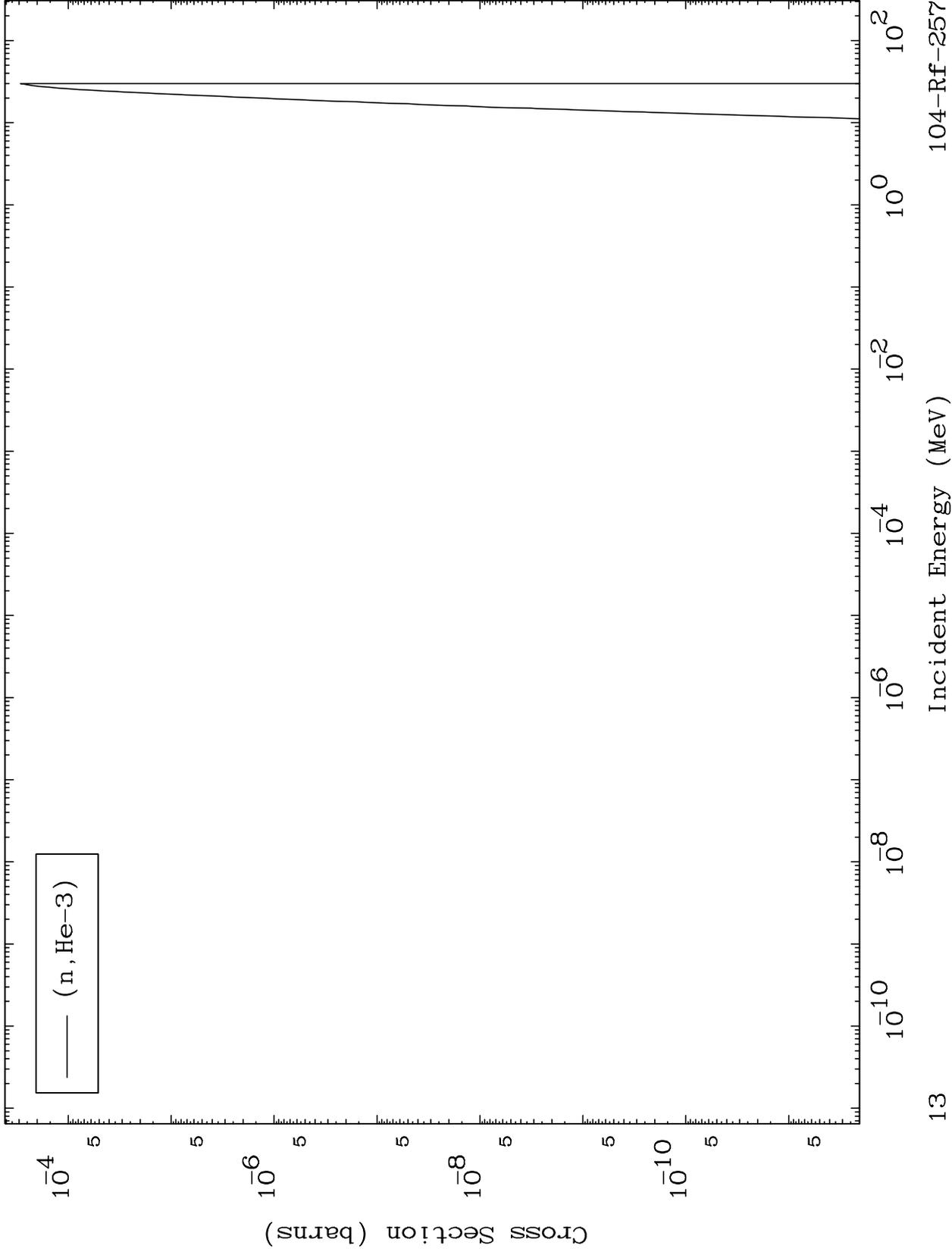
104-Rf-257

12

MAT 457

(n,He3) Levels
293 Kelvin Cross Sections

104-Rf-257



13

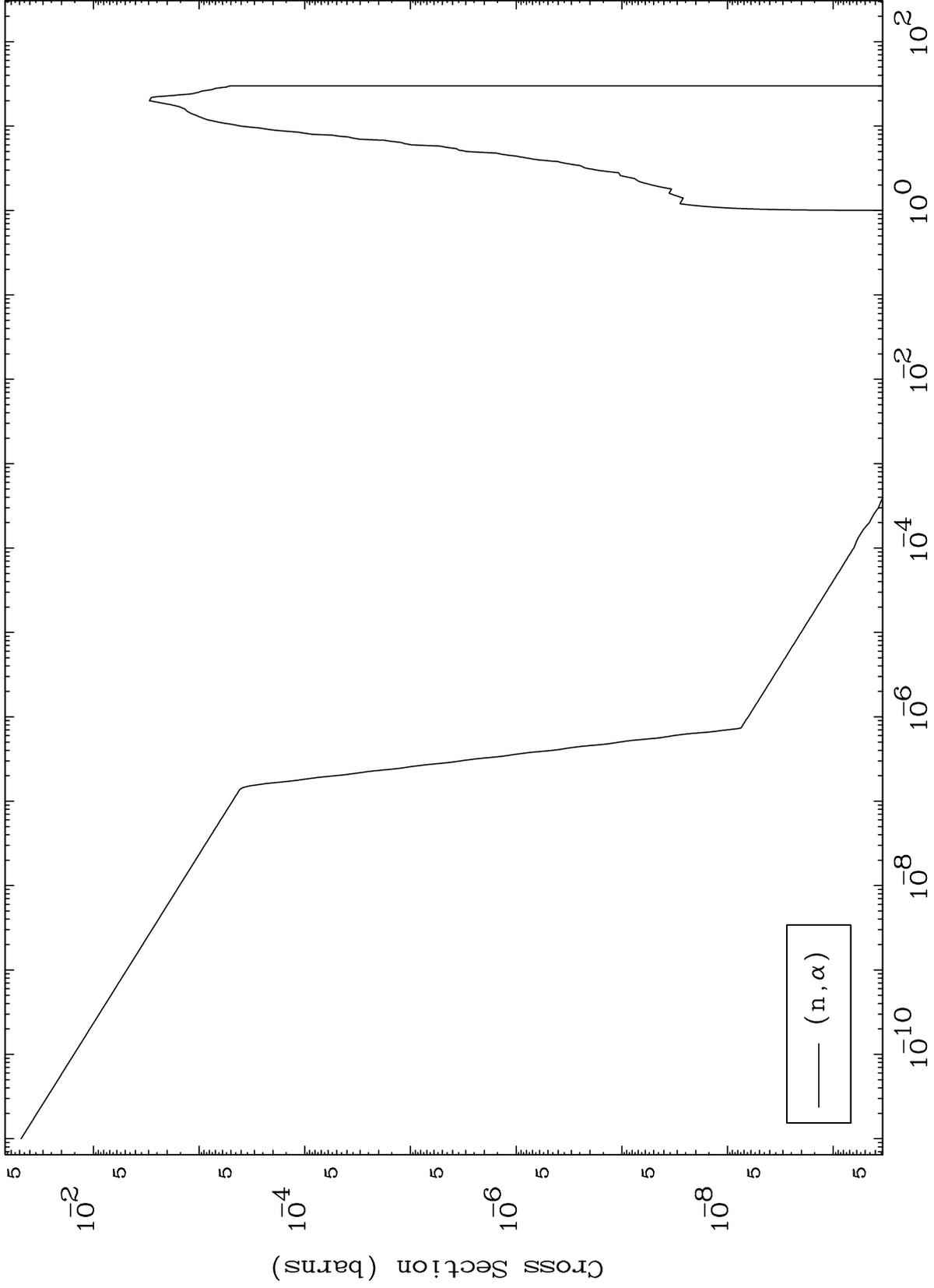
Incident Energy (MeV)

104-Rf-257

MAT 457

(n,α) Levels
293 Kelvin Cross Sections

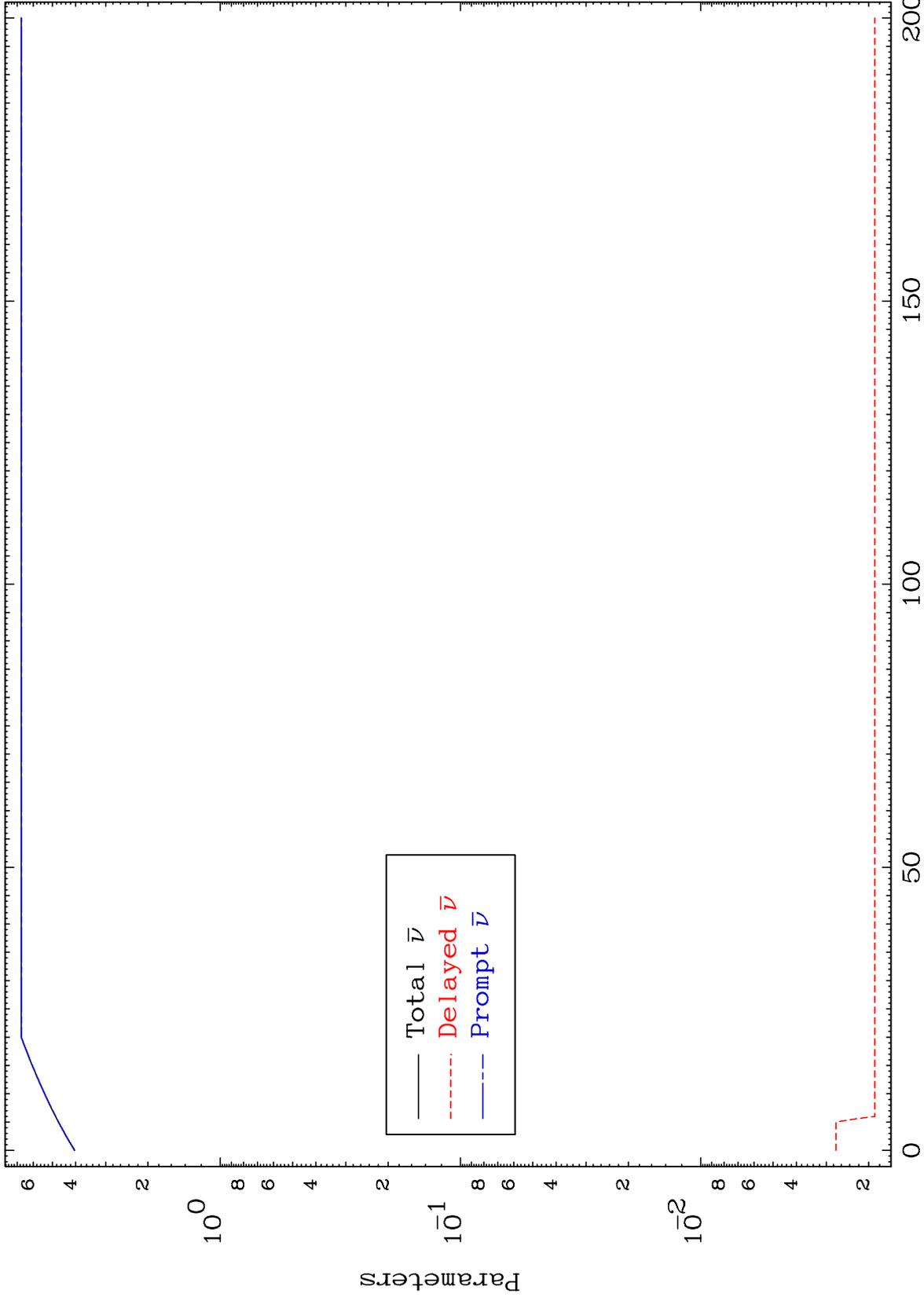
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MAT 457

Energy Release
Parameters

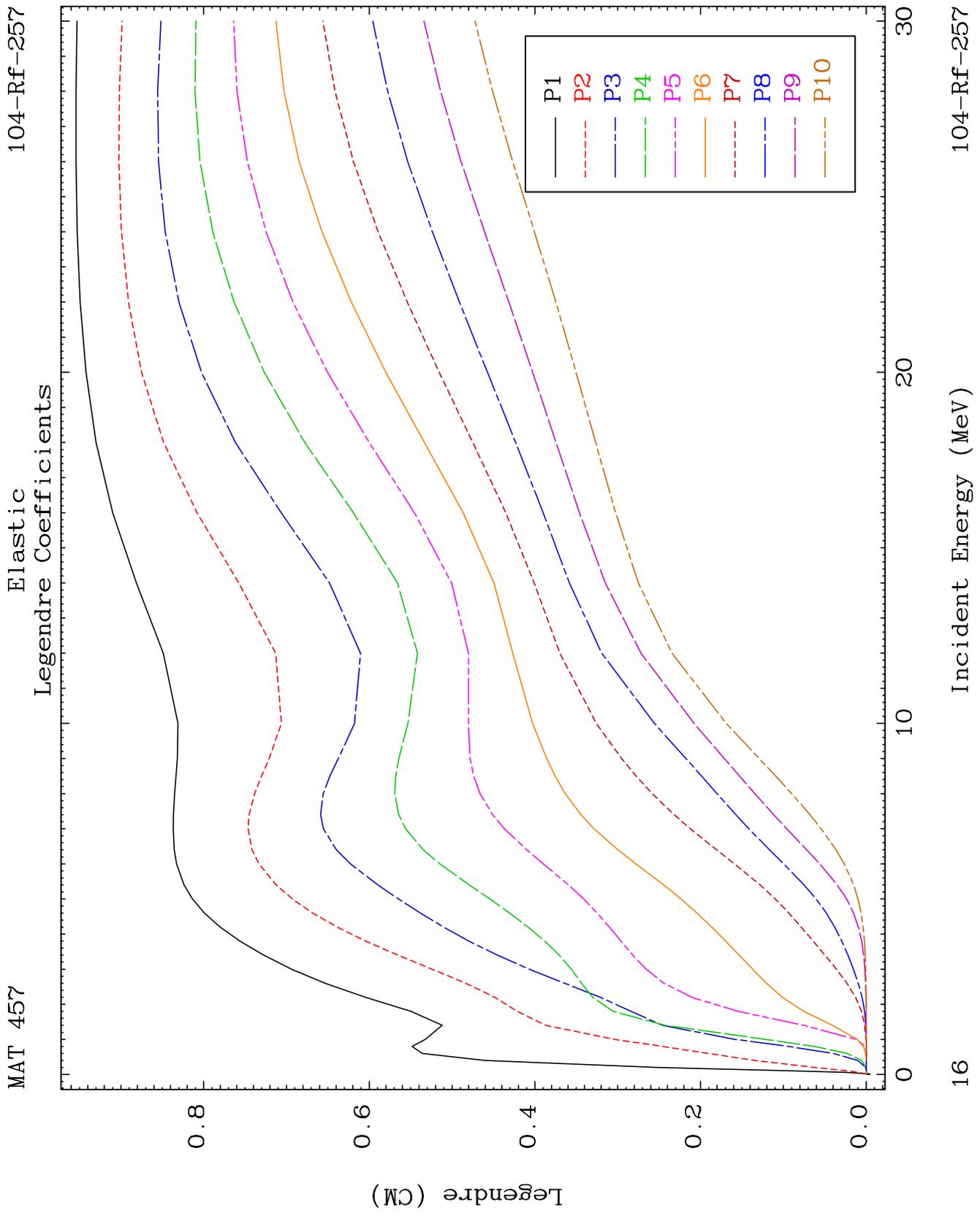
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15

Incident Energy (MeV)

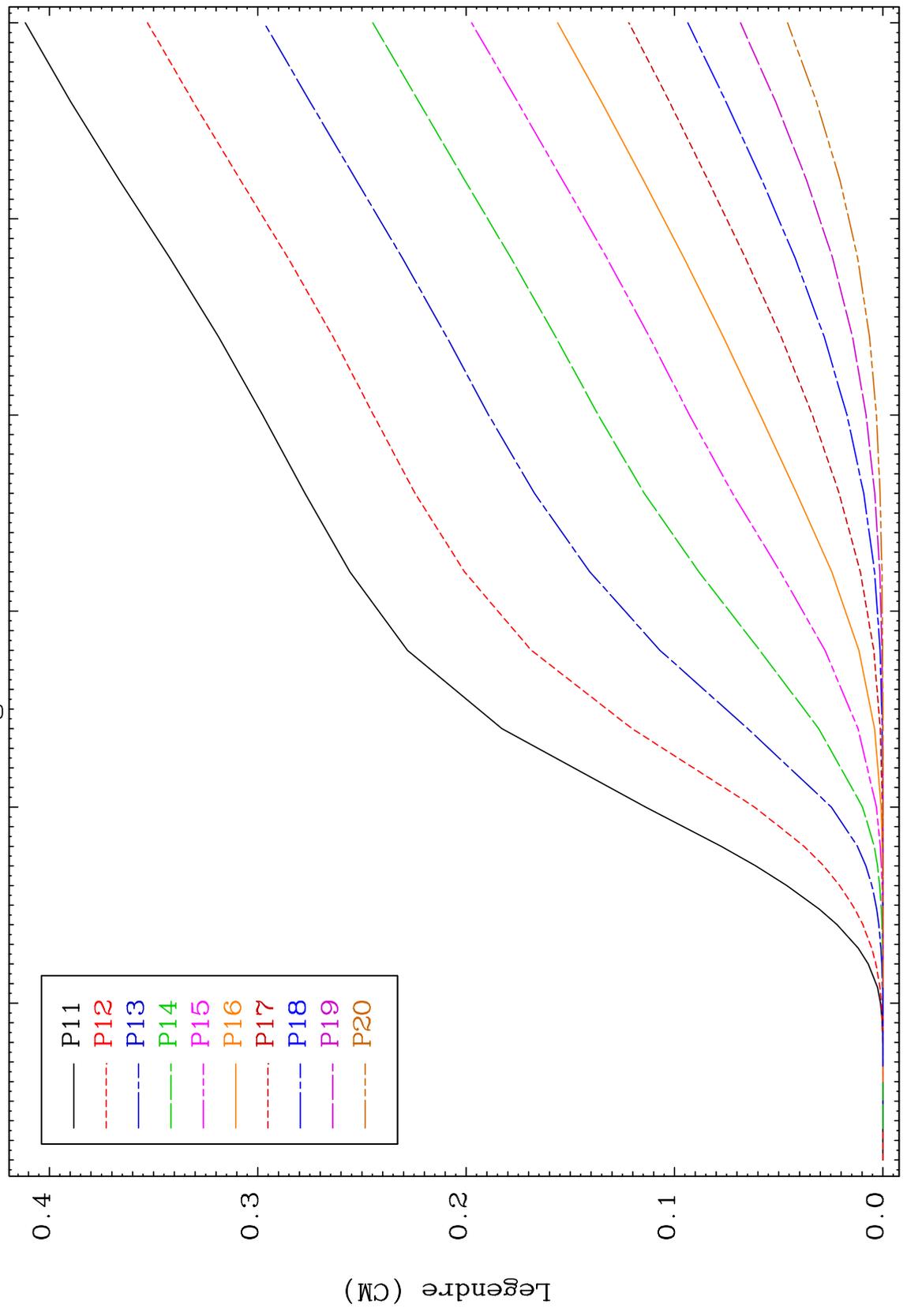
104-Rf-257



MAT 457

Elastic Legendre Coefficients

104-Rf-257



17

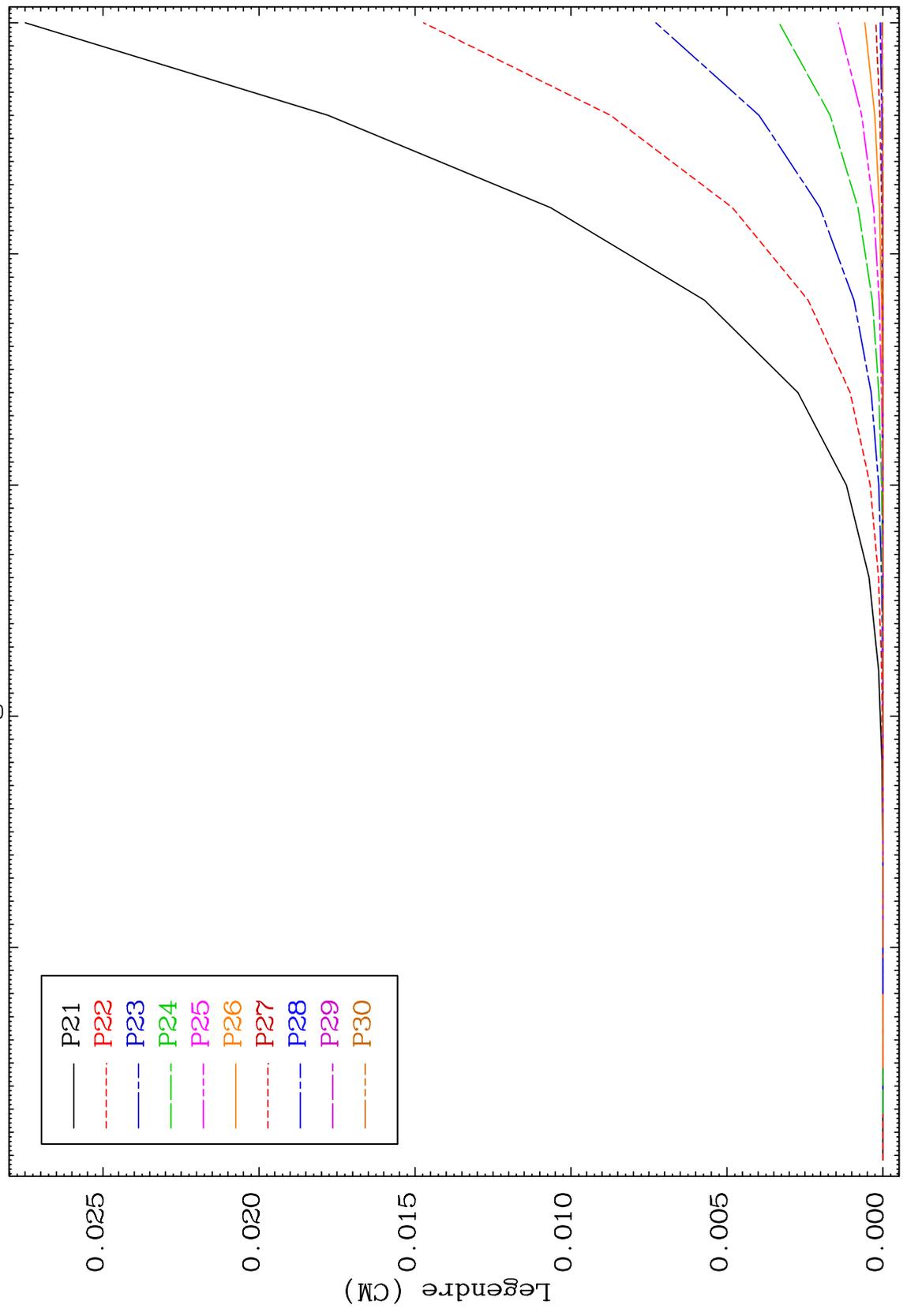
Incident Energy (MeV)

104-Rf-257

MAT 457

Elastic Legendre Coefficients

104-Rf-257



18

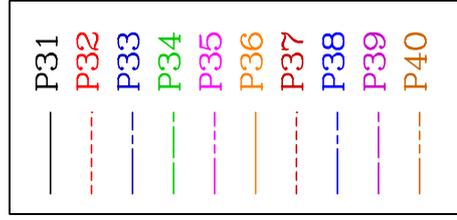
Incident Energy (MeV)

104-Rf-257

MAT 457

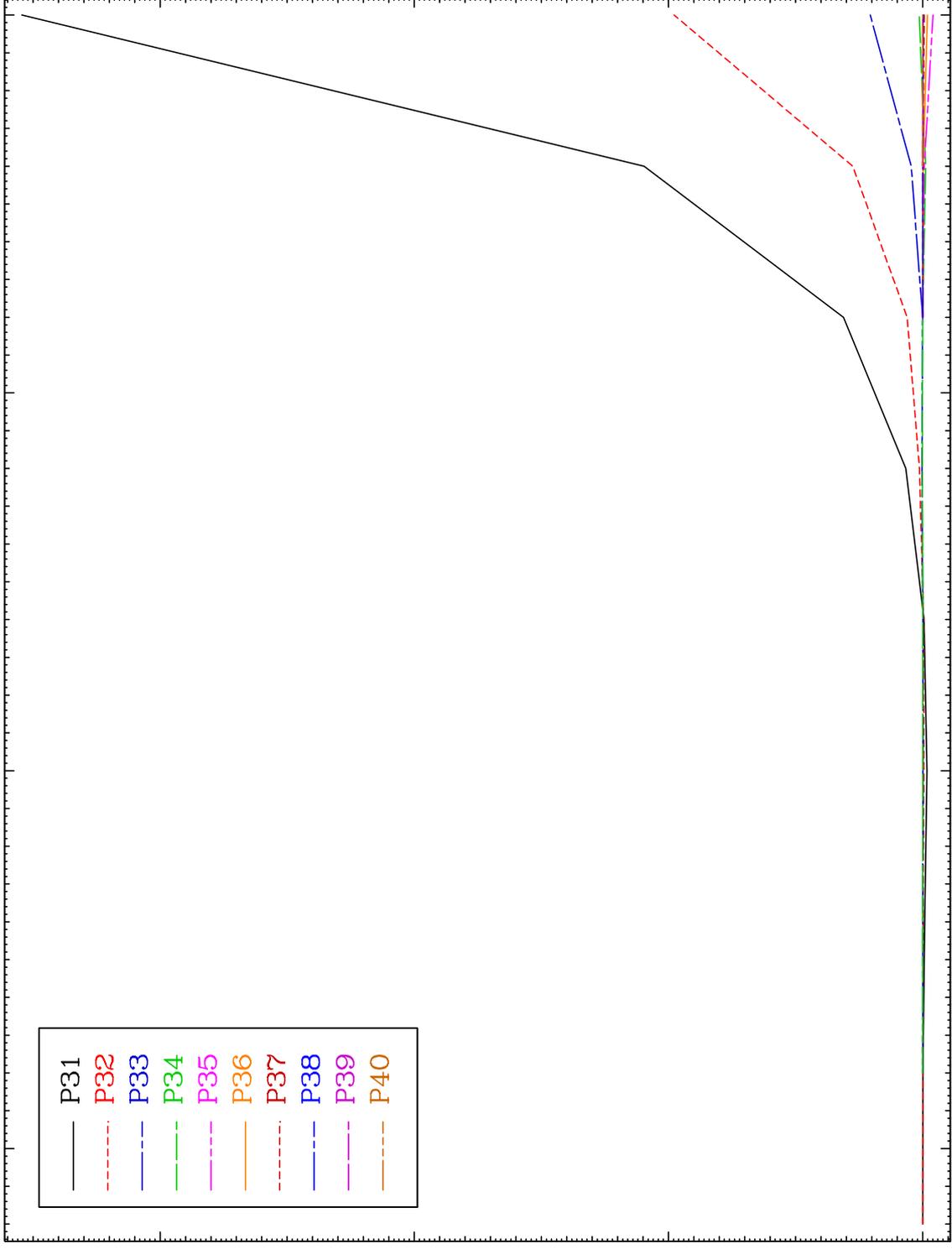
Elastic Legendre Coefficients

104-Rf-257



$\times 10^{-6}$

Legendre (CM)



15

20

25

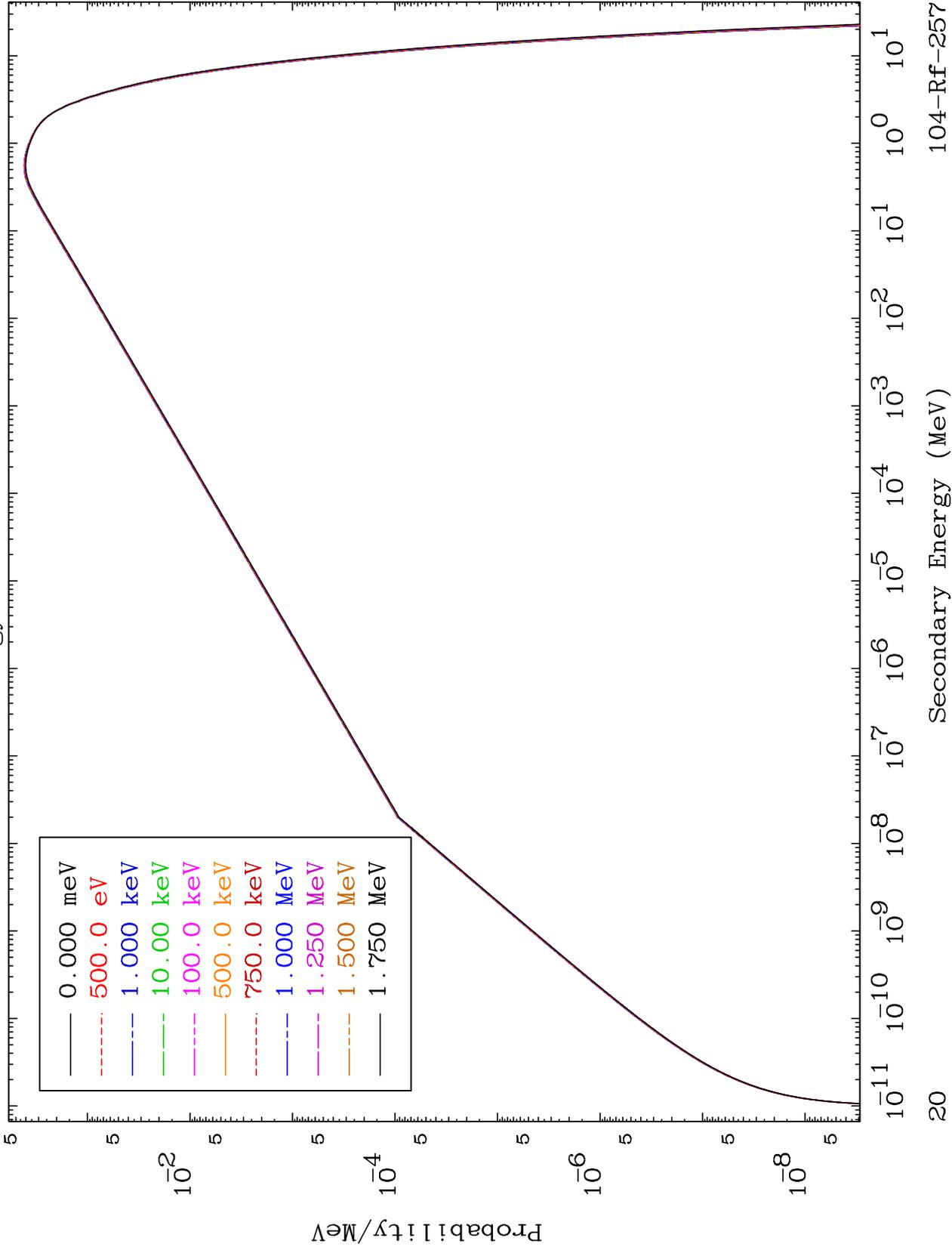
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19

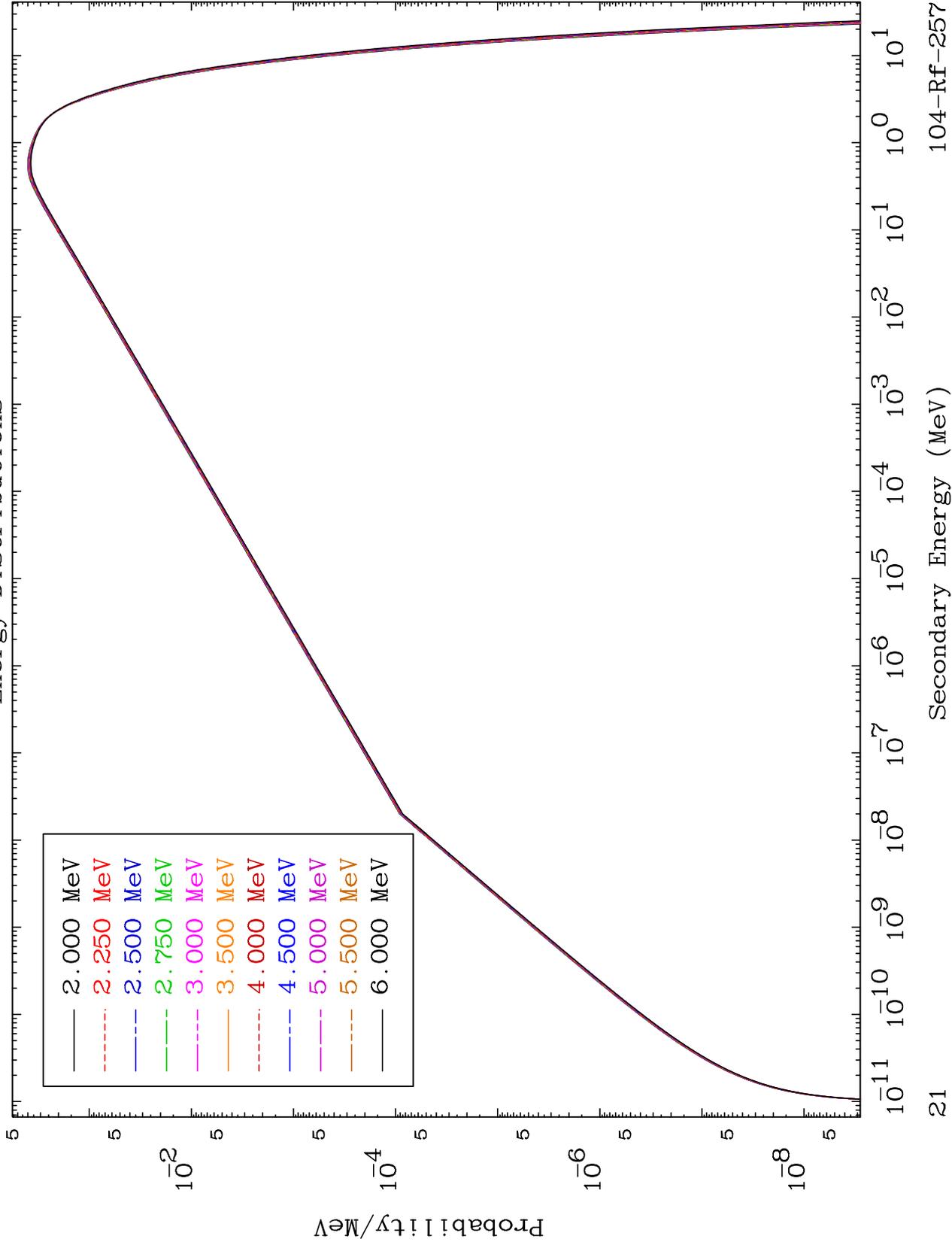
Incident Energy (MeV)

104-Rf-257

Fission Energy Distributions



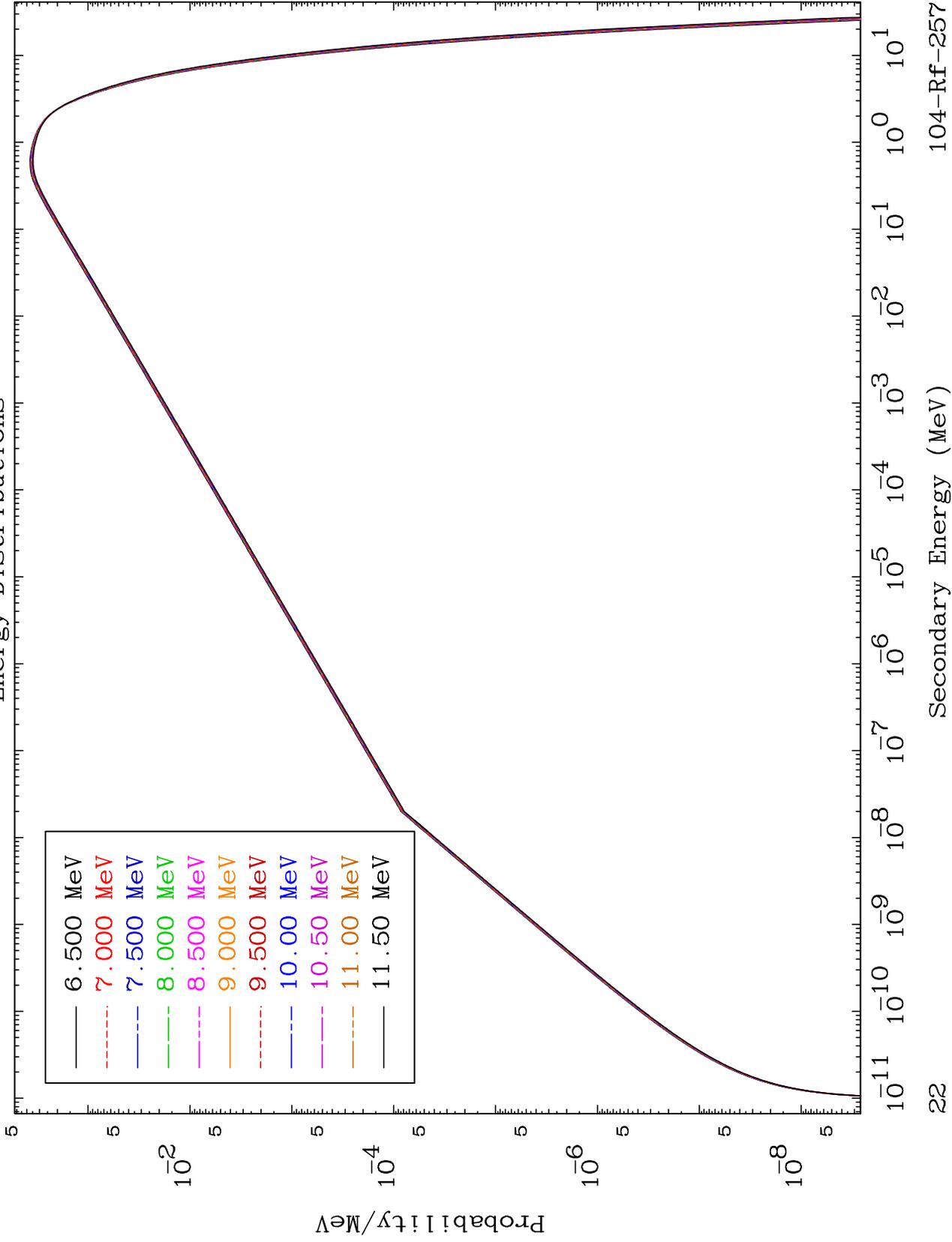
Fission Energy Distributions



MAT 457

Fission Energy Distributions

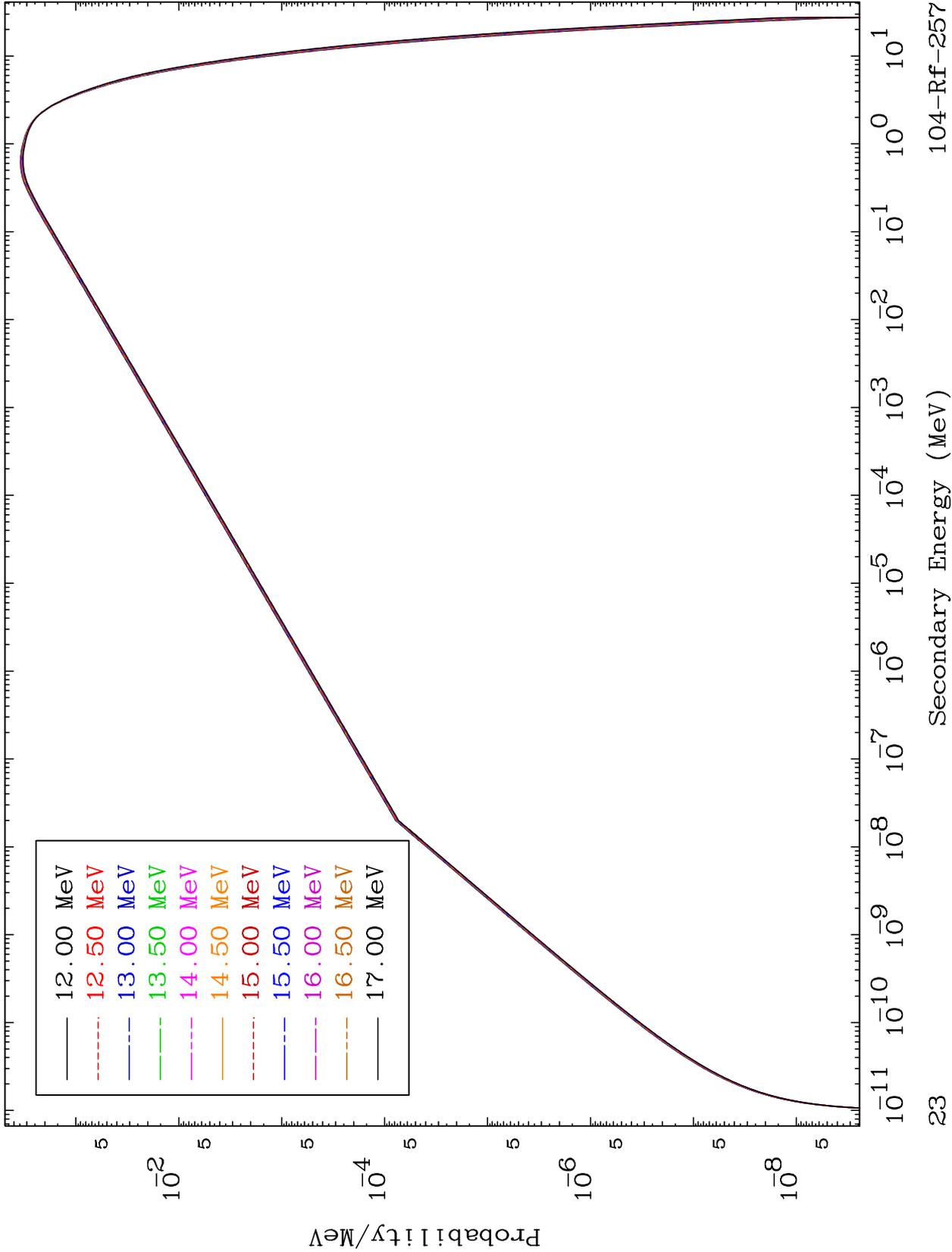
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Fission Energy Distributions

104-Rf-257



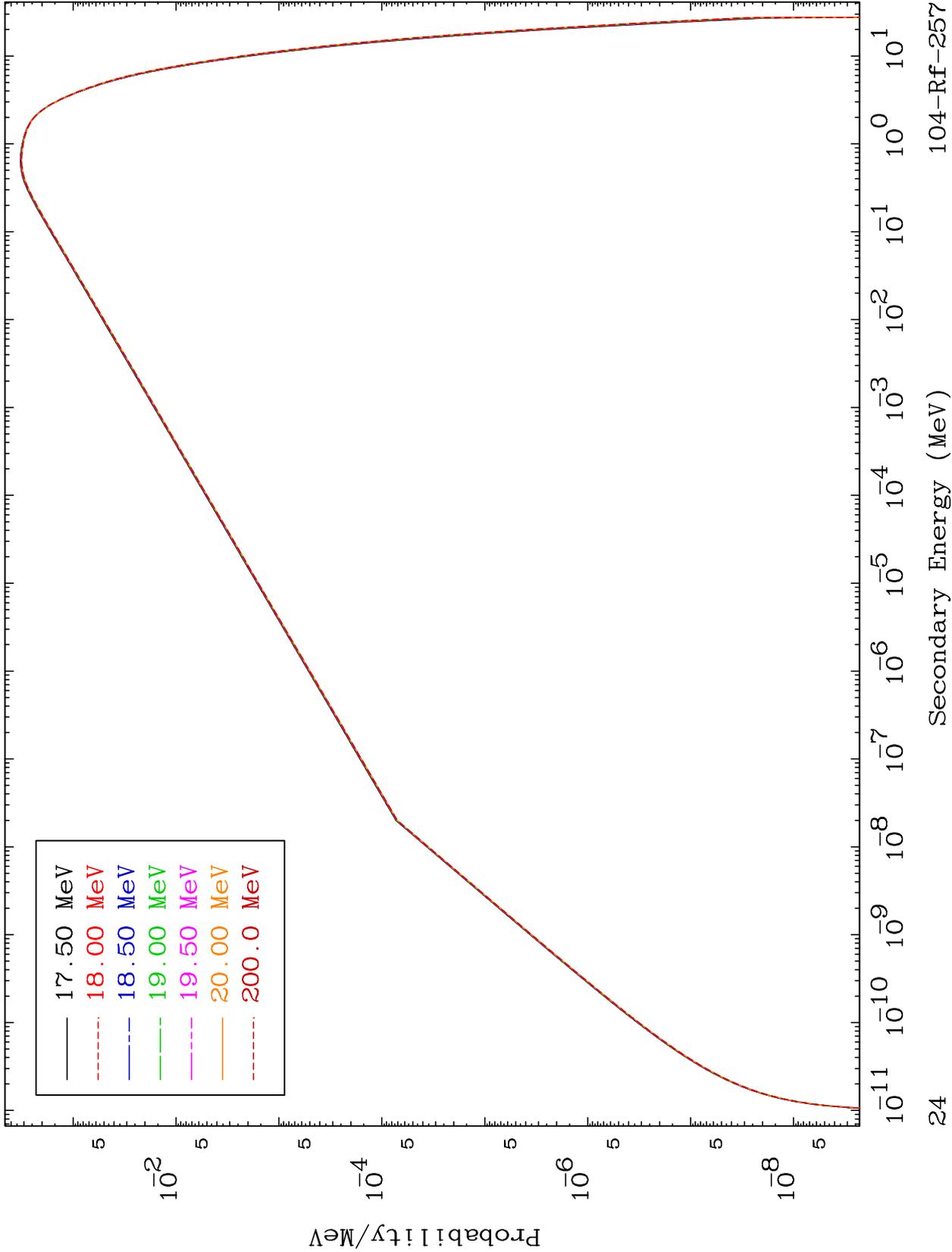
104-Rf-257

Secondary Energy (MeV)

MAT 457

Fission Energy Distributions

104-Rf-257



104-Rf-257

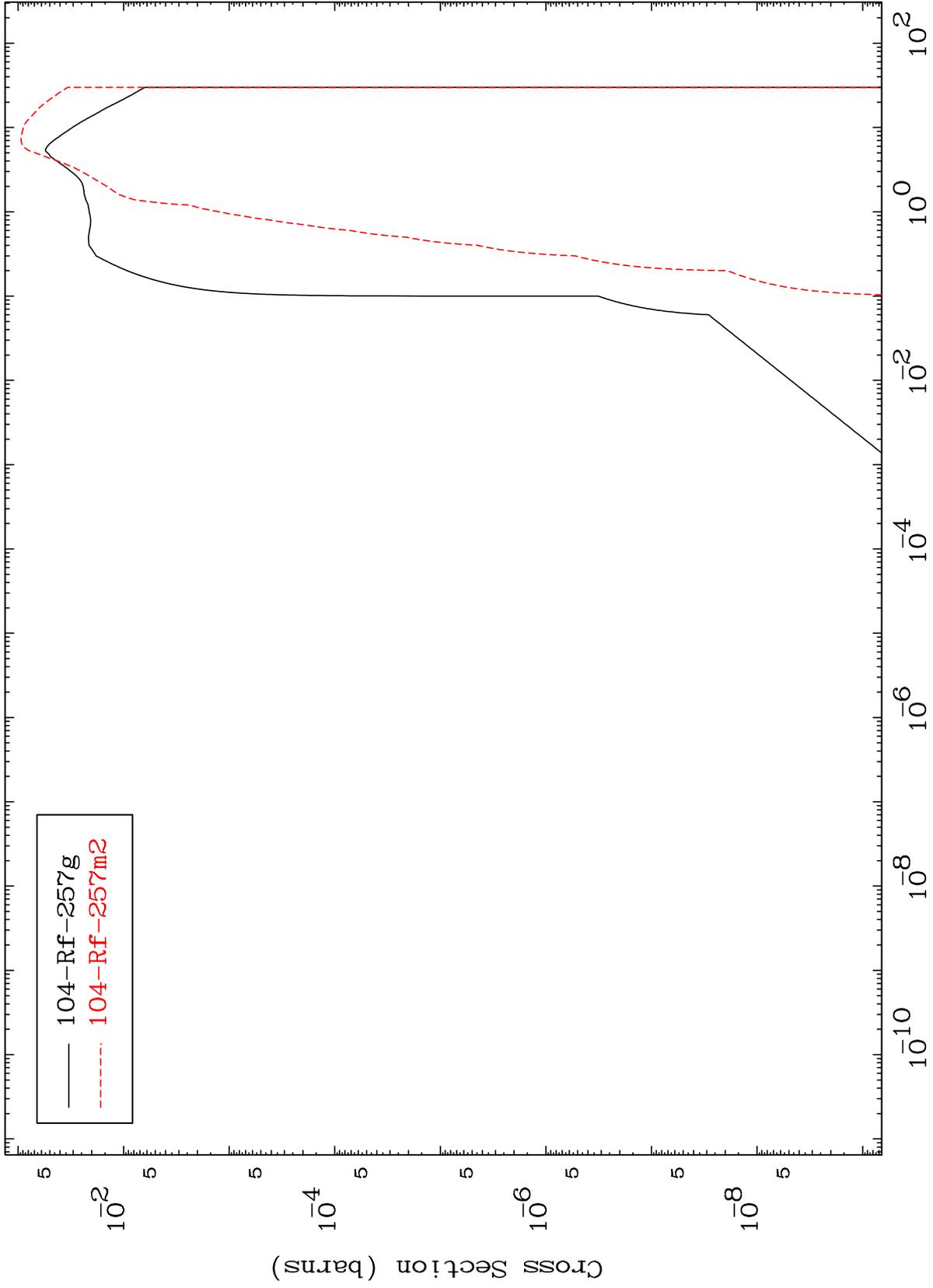
Secondary Energy (MeV)

24

MAT 457

Radionuclide Production Cross Section

104-Rf-257



— 104-Rf-257g
- - - 104-Rf-257m2

25

Incident Energy (MeV)

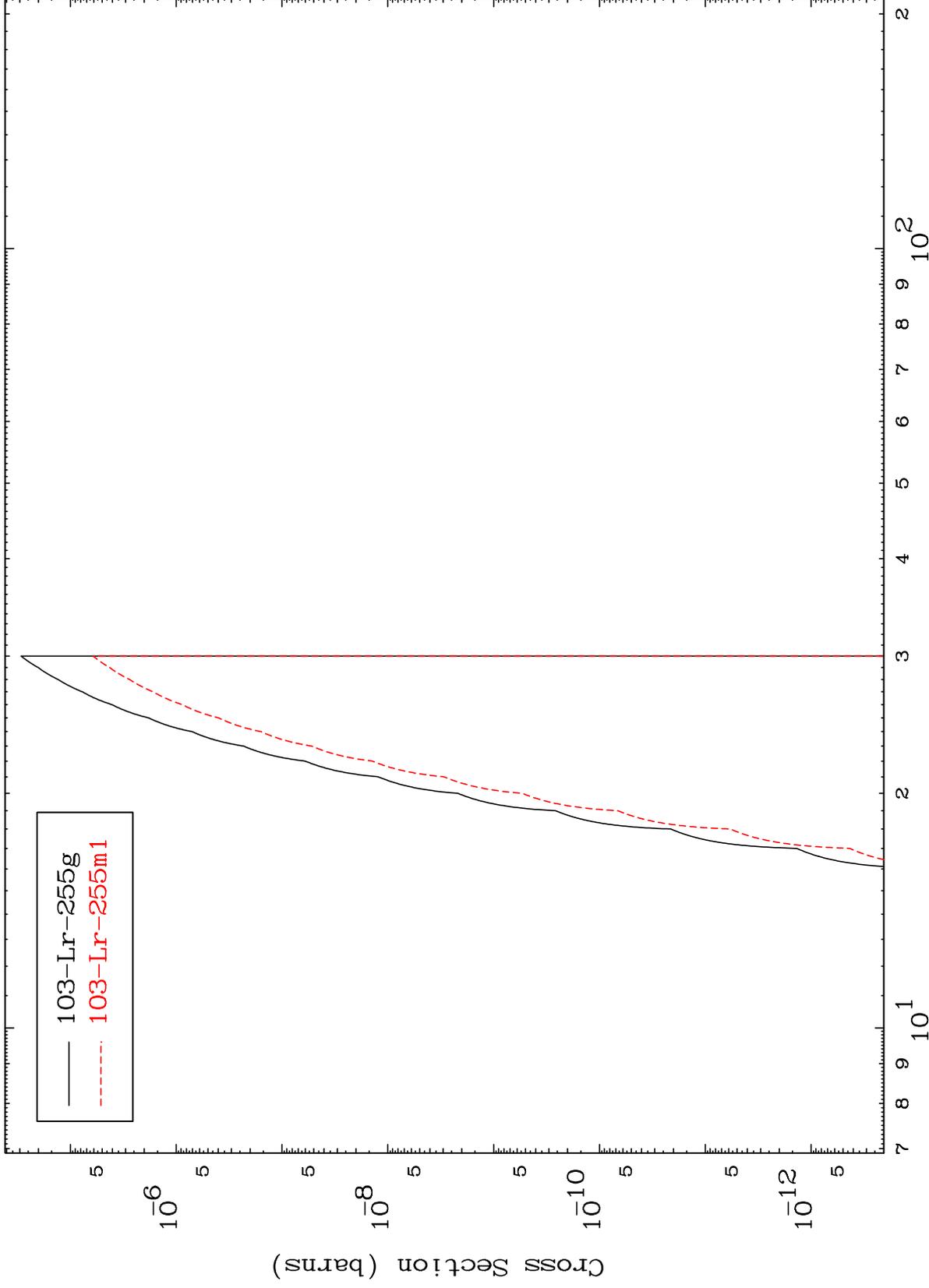
104-Rf-257

MAT 457

(n,n') d

104-Rf-257

Radionuclide Production Cross Section



26

Incident Energy (MeV)

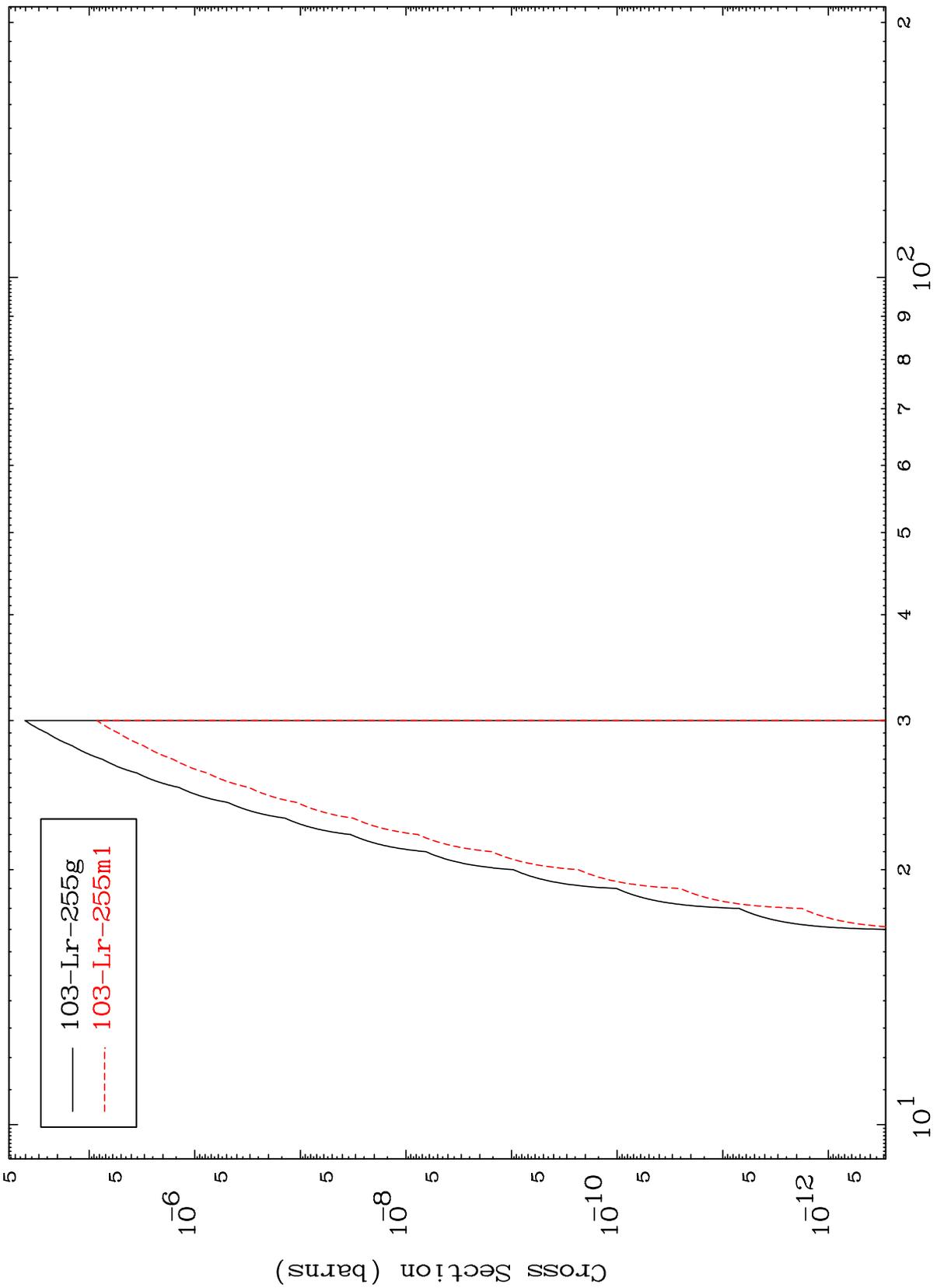
104-Rf-257

MAT 457

(n,2n) p

104-Rf-257

Radionuclide Production Cross Section



Legend:
— 103-Lr-255g
- - - 103-Lr-255m1

27

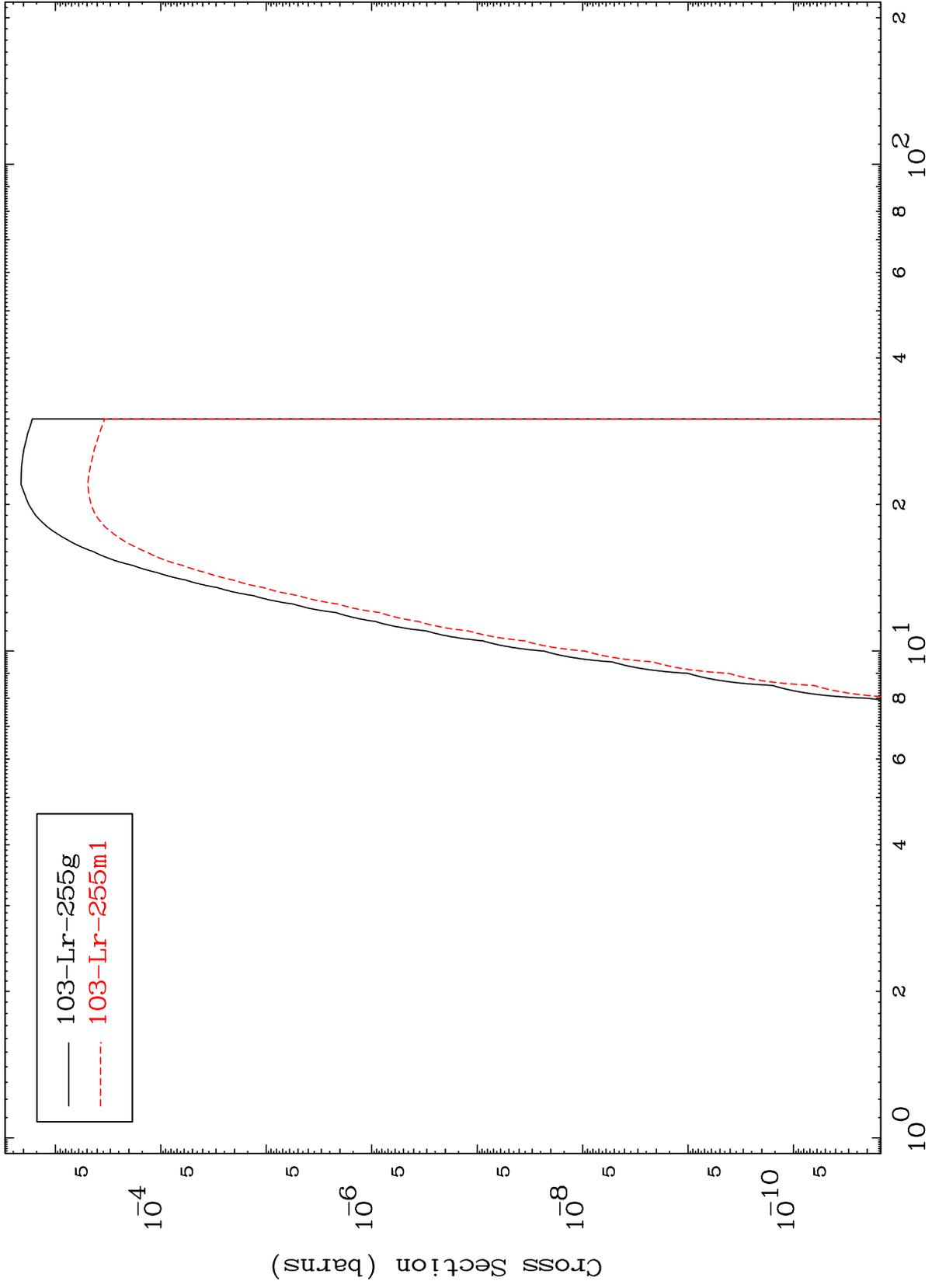
Incident Energy (MeV)

104-Rf-257

MAT 457

104-Rf-257

(n, t)
Radionuclide Production Cross Section



28

Incident Energy (MeV)

104-Rf-257