

Program EVALPLOT  
(Version 2021-1)

by

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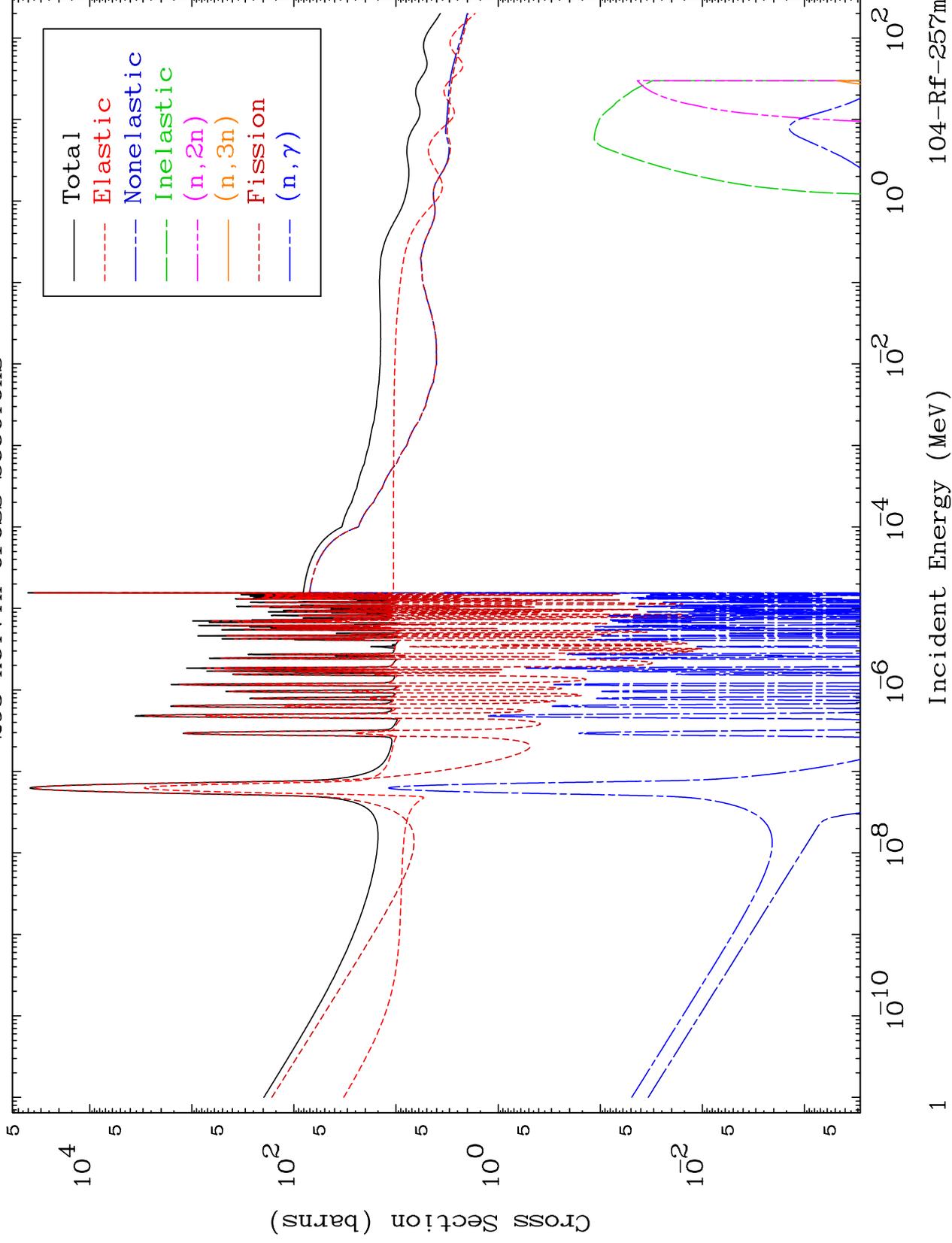
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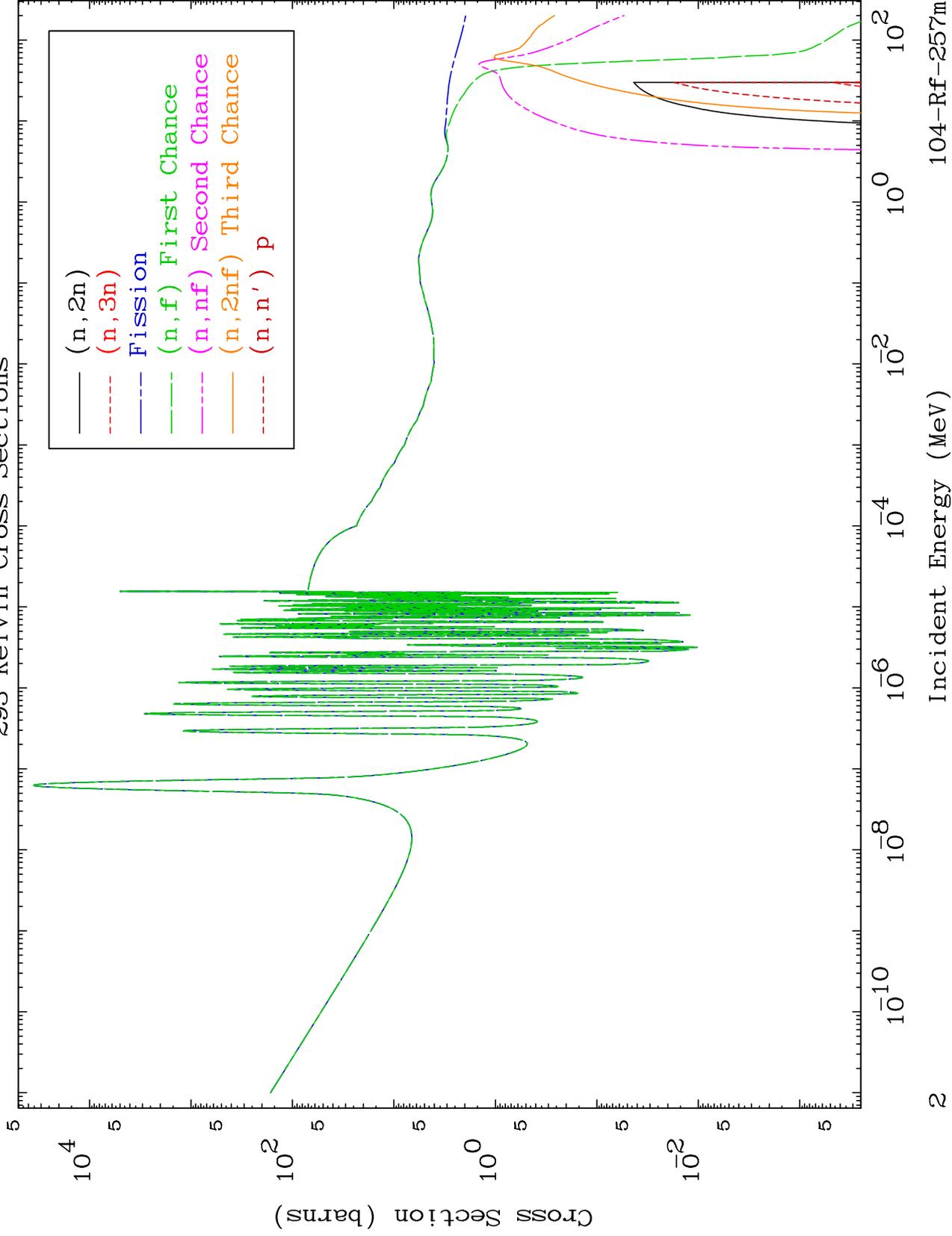
Press Mouse Button to Start

MAT 457

Neutron Major  
293 Kelvin Cross Sections

104-Rf-257m

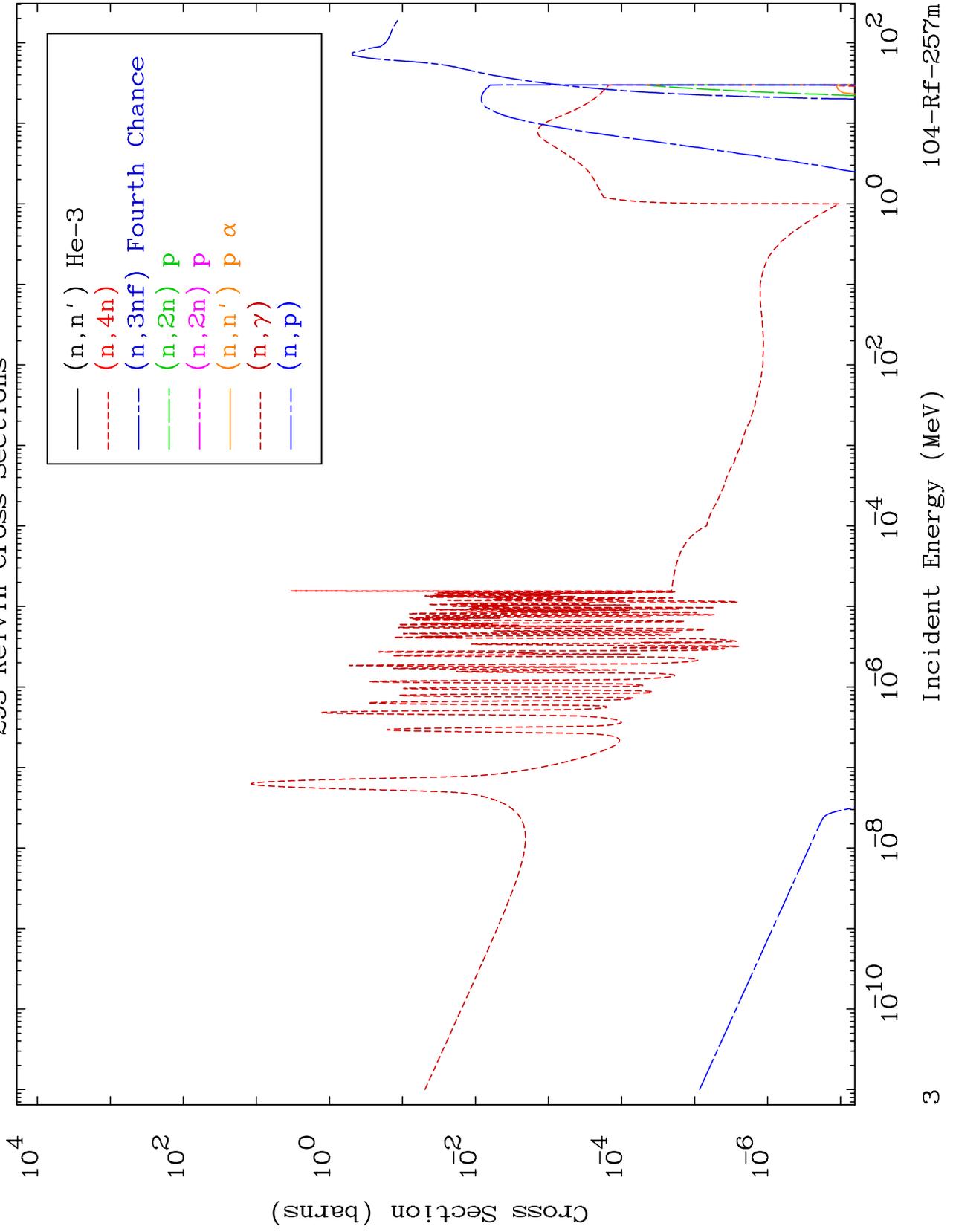




MAT 457

Neutron Absorption  
293 Kelvin Cross Sections

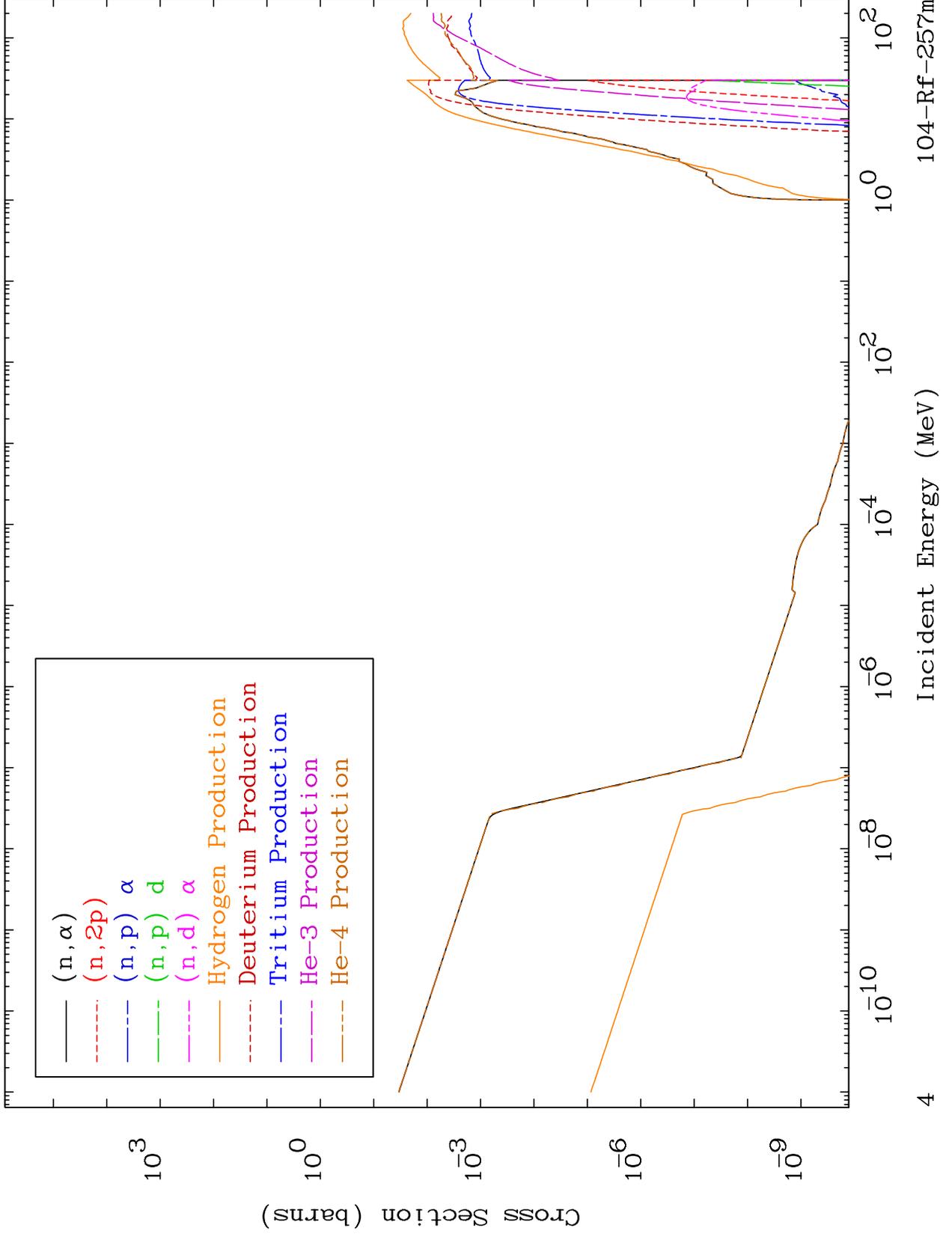
104-Rf-257m



MAT 457

Neutron Absorption  
293 Kelvin Cross Sections

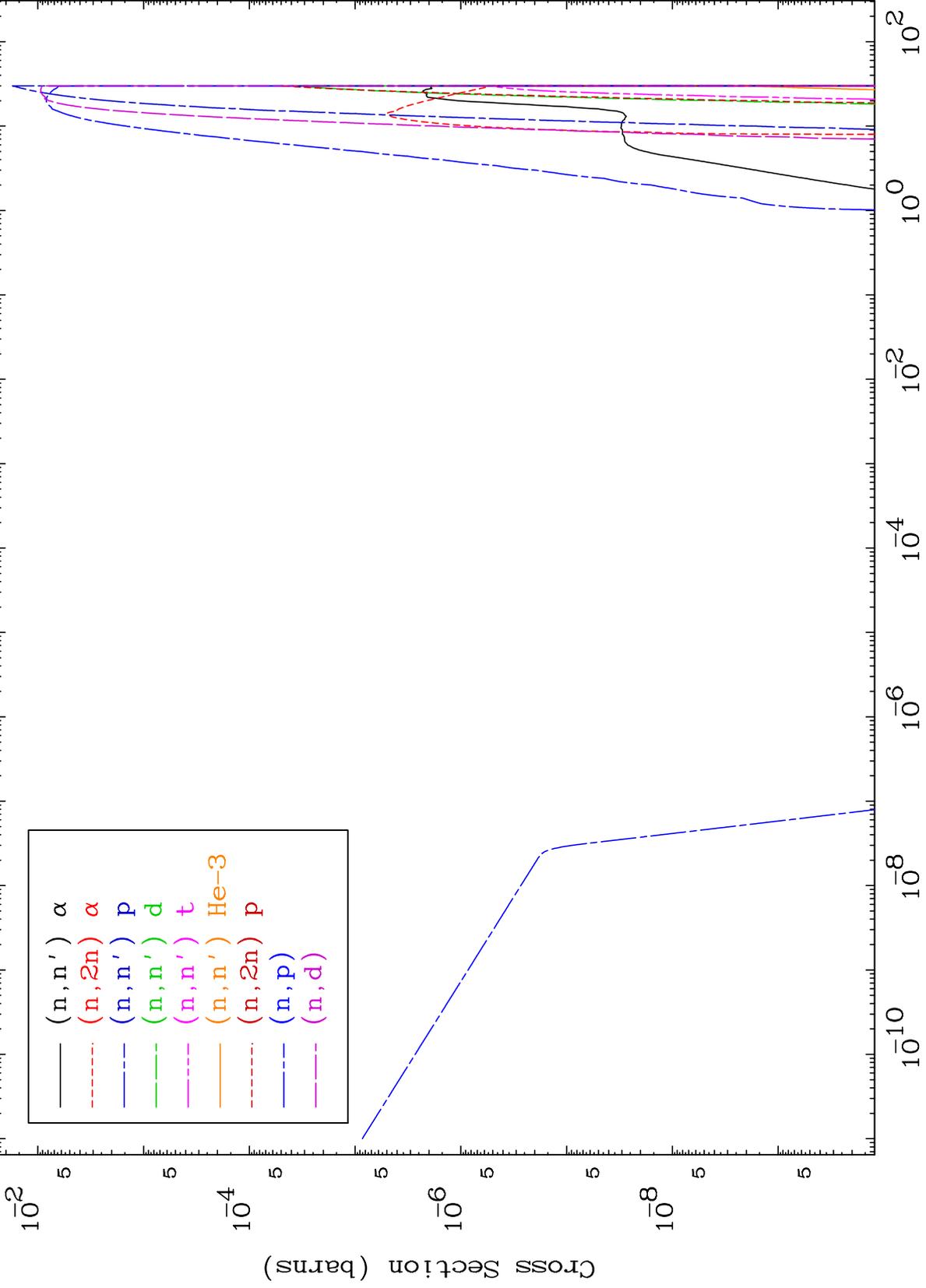
104-Rf-257m



MAT 457

Charged Particle  
293 Kelvin Cross Sections

104-Rf-257m



5

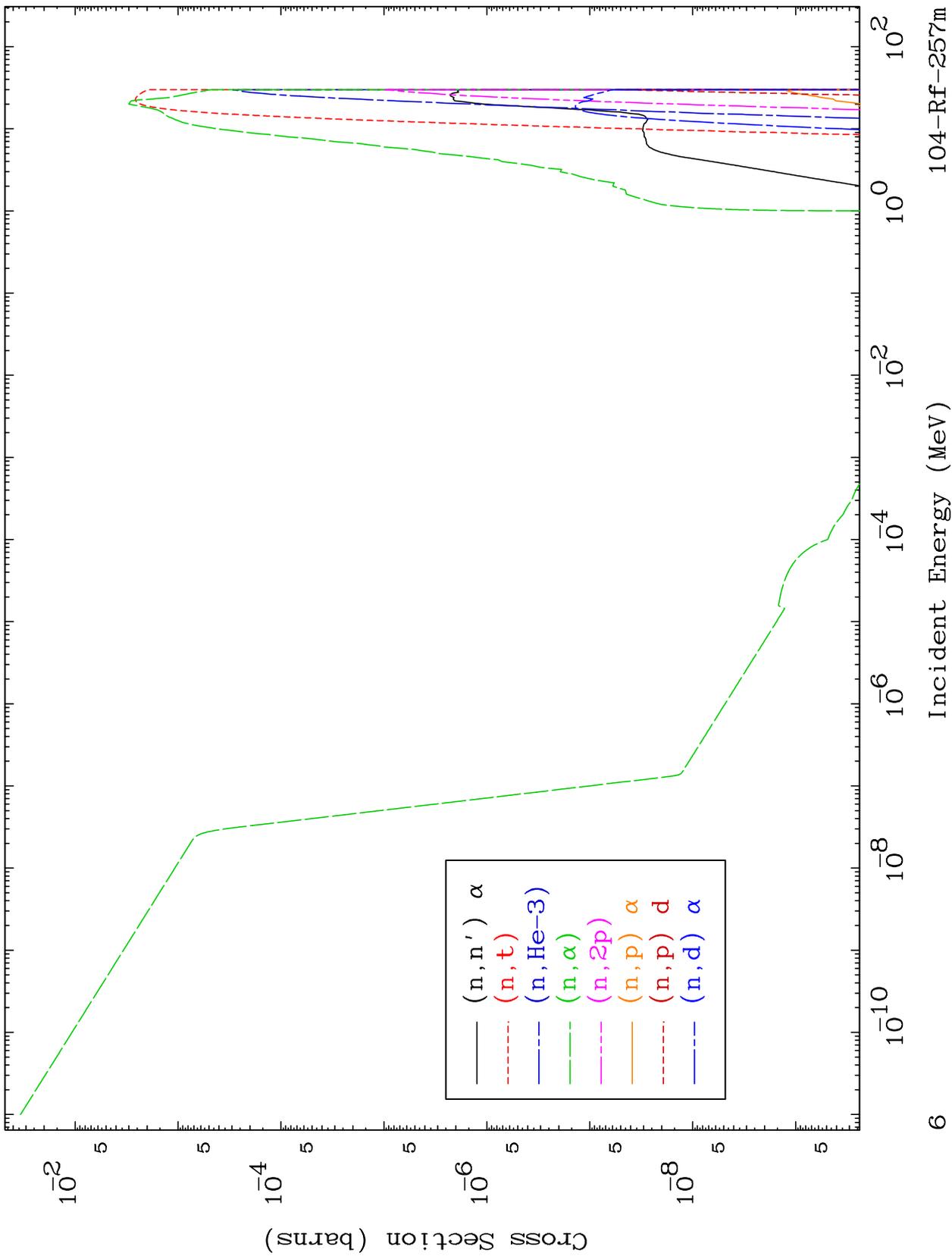
Incident Energy (MeV)

104-Rf-257m

MAT 457

Charged Particle  
293 Kelvin Cross Sections

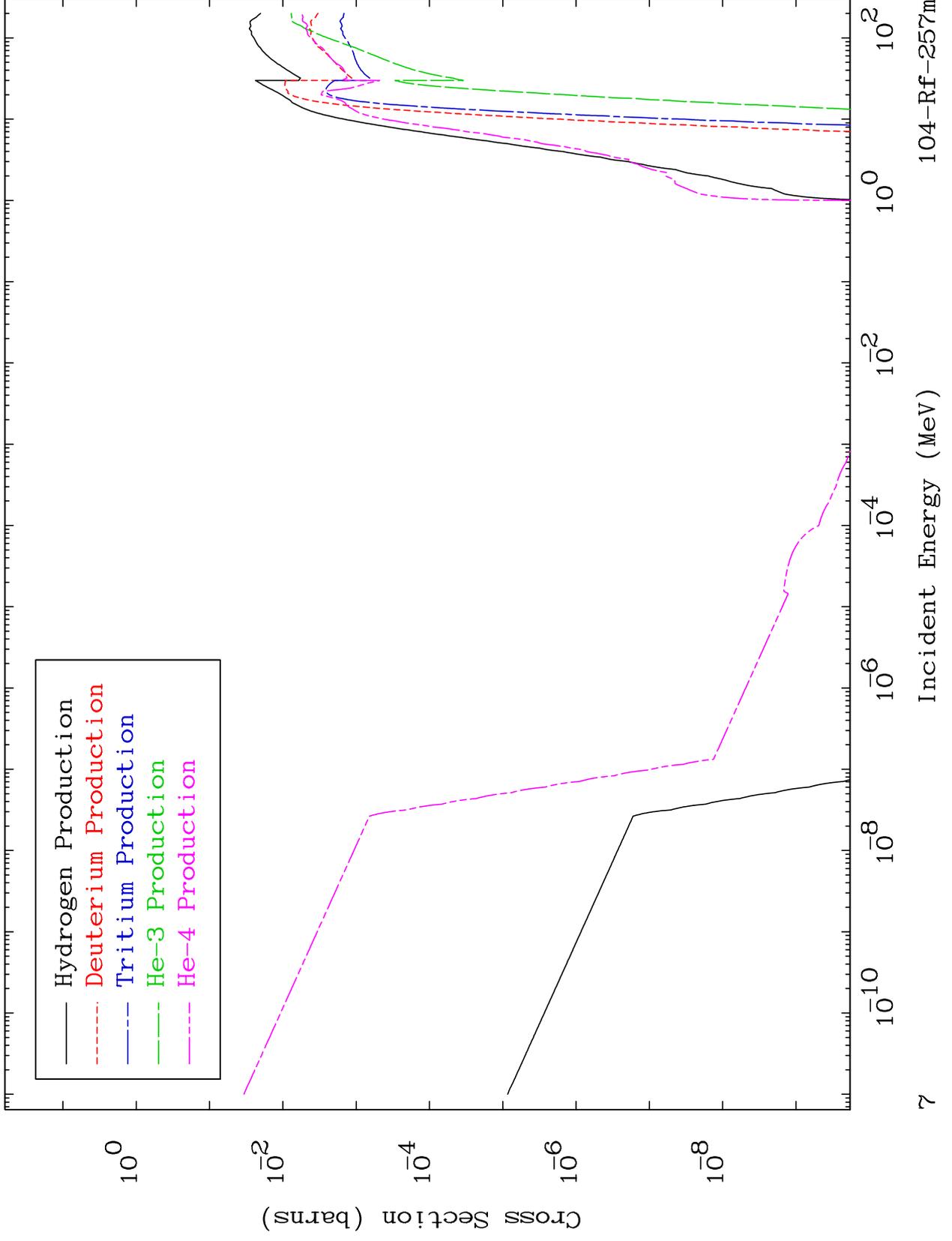
104-Rf-257m



MAT 457

Particle Production  
293 Kelvin Cross Sections

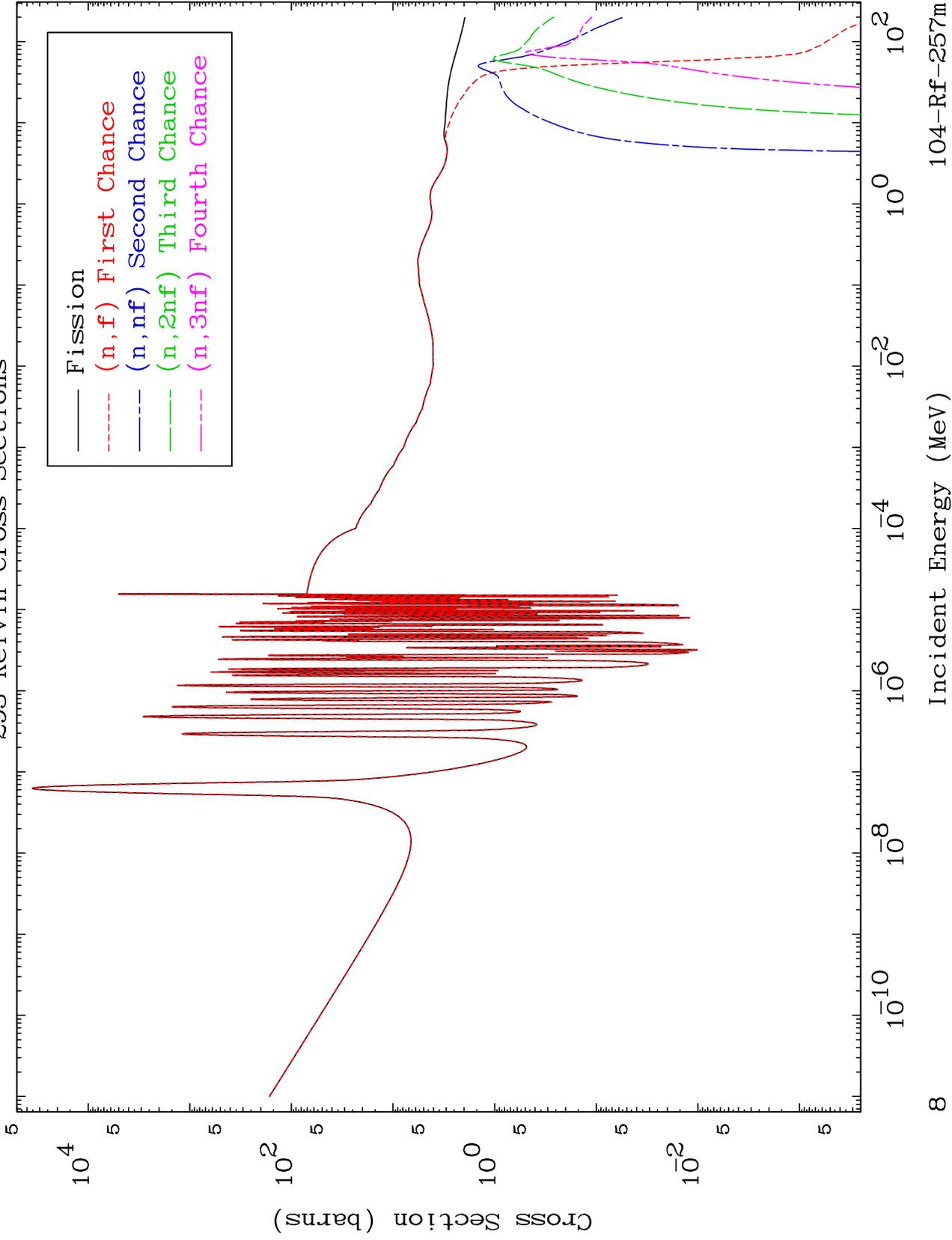
104-Rf-257m



MAT 457

Fission  
293 Kelvin Cross Sections

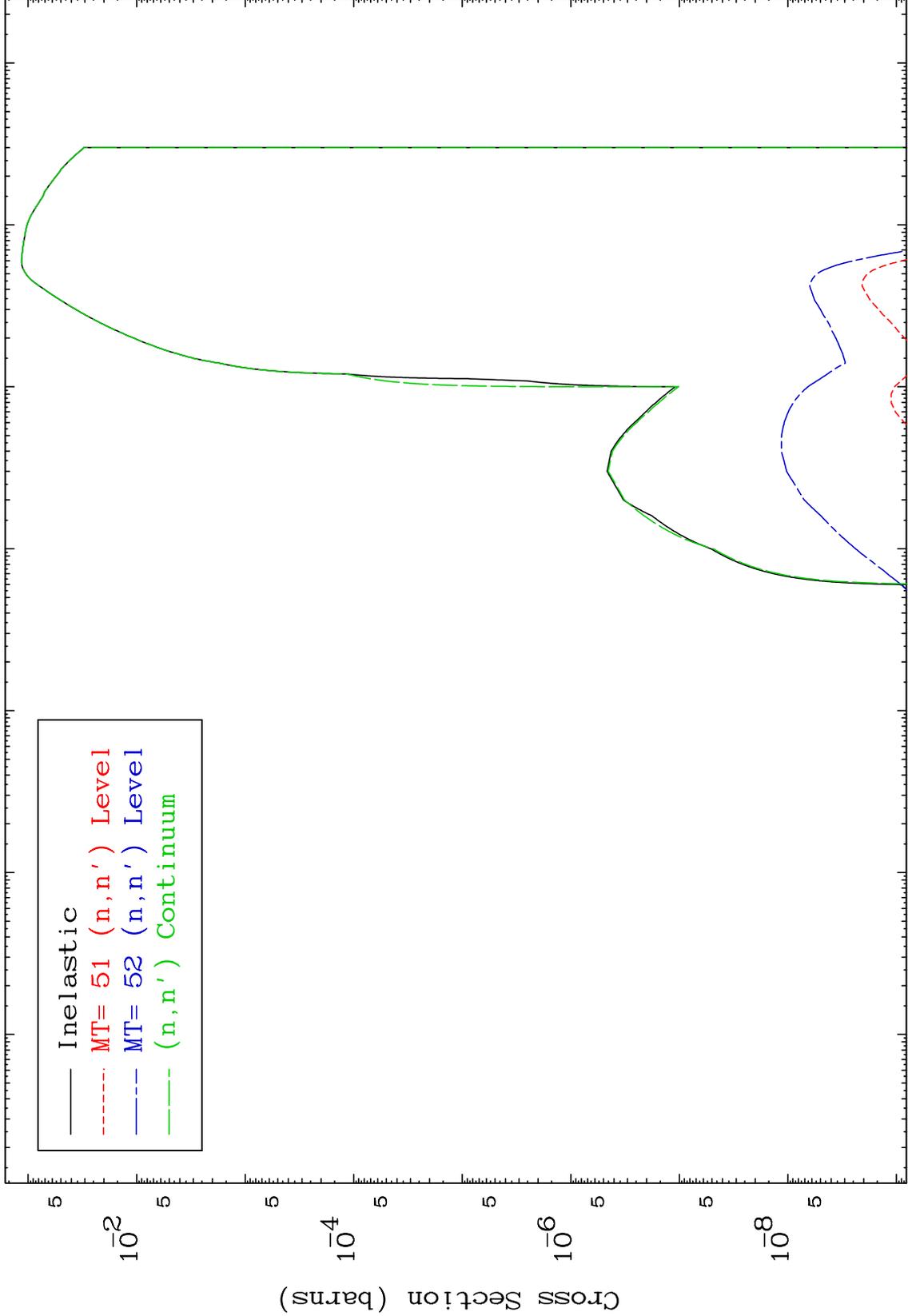
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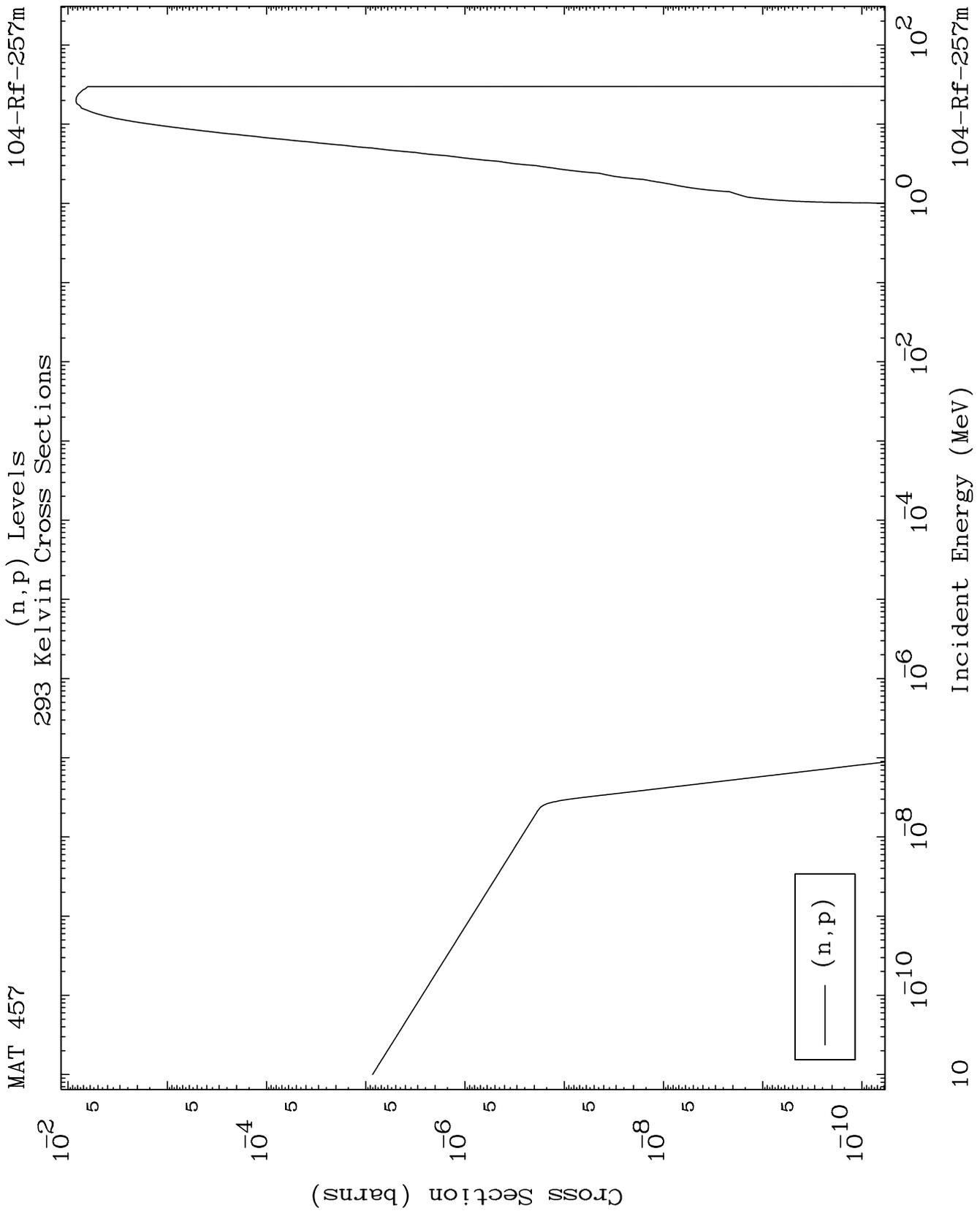


MAT 457

(n,n') Levels  
293 Kelvin Cross Sections

104-Rf-257m

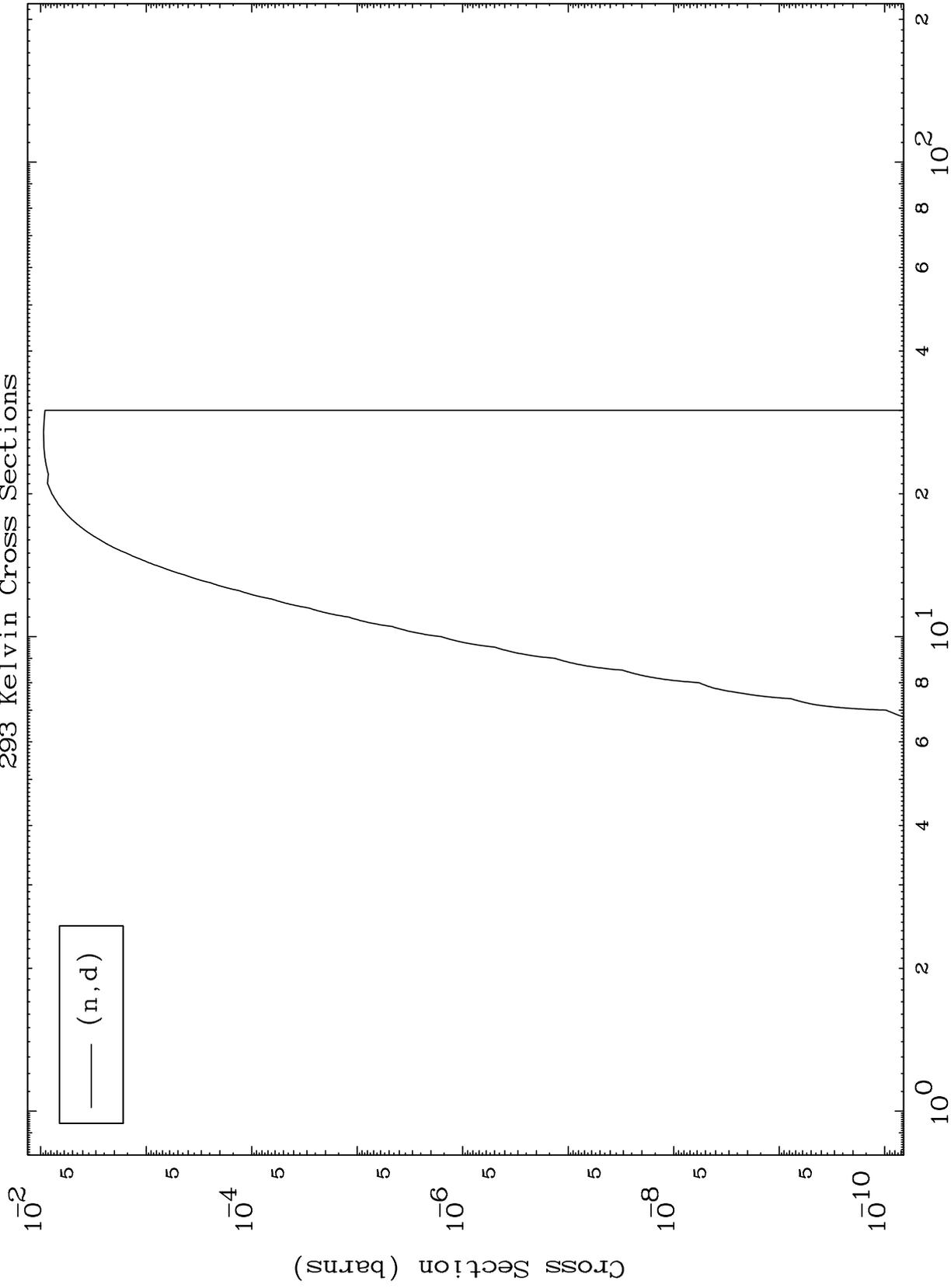




MAT 457

(n,d) Levels  
293 Kelvin Cross Sections

104-Rf-257m



11

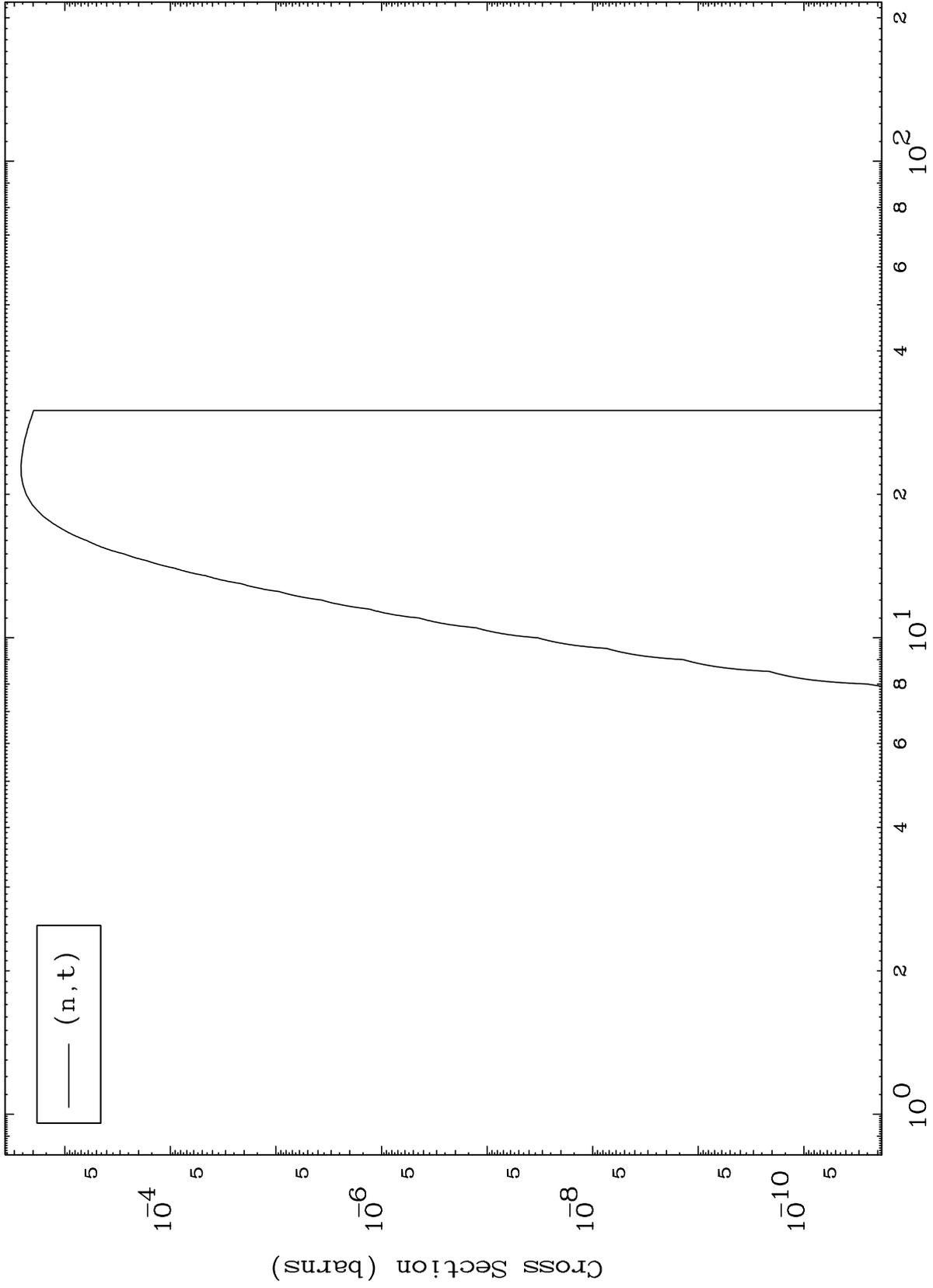
Incident Energy (MeV)

104-Rf-257m

MAT 457

(n, t) Levels  
293 Kelvin Cross Sections

104-Rf-257m



12

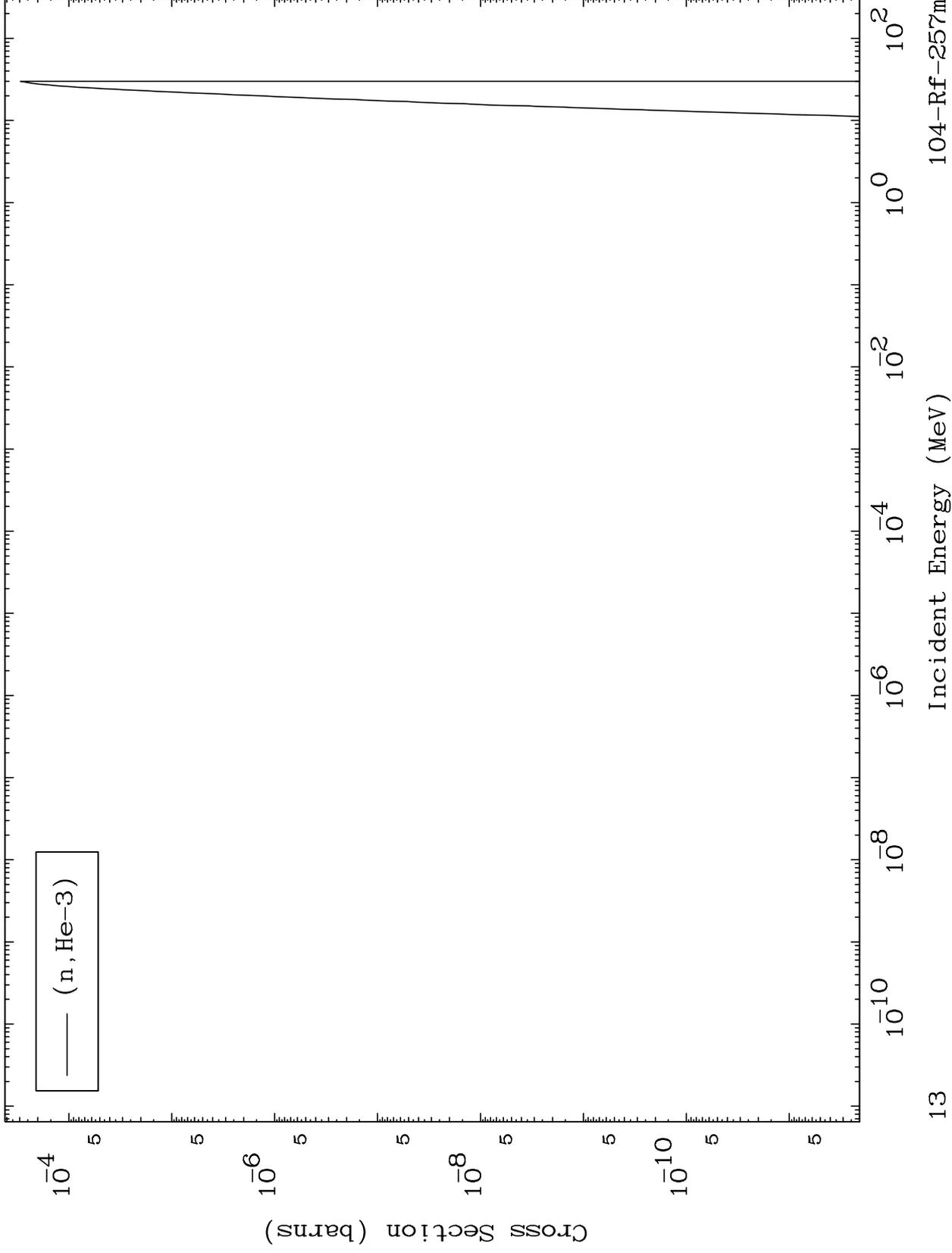
Incident Energy (MeV)

104-Rf-257m

MAT 457

(n,He3) Levels  
293 Kelvin Cross Sections

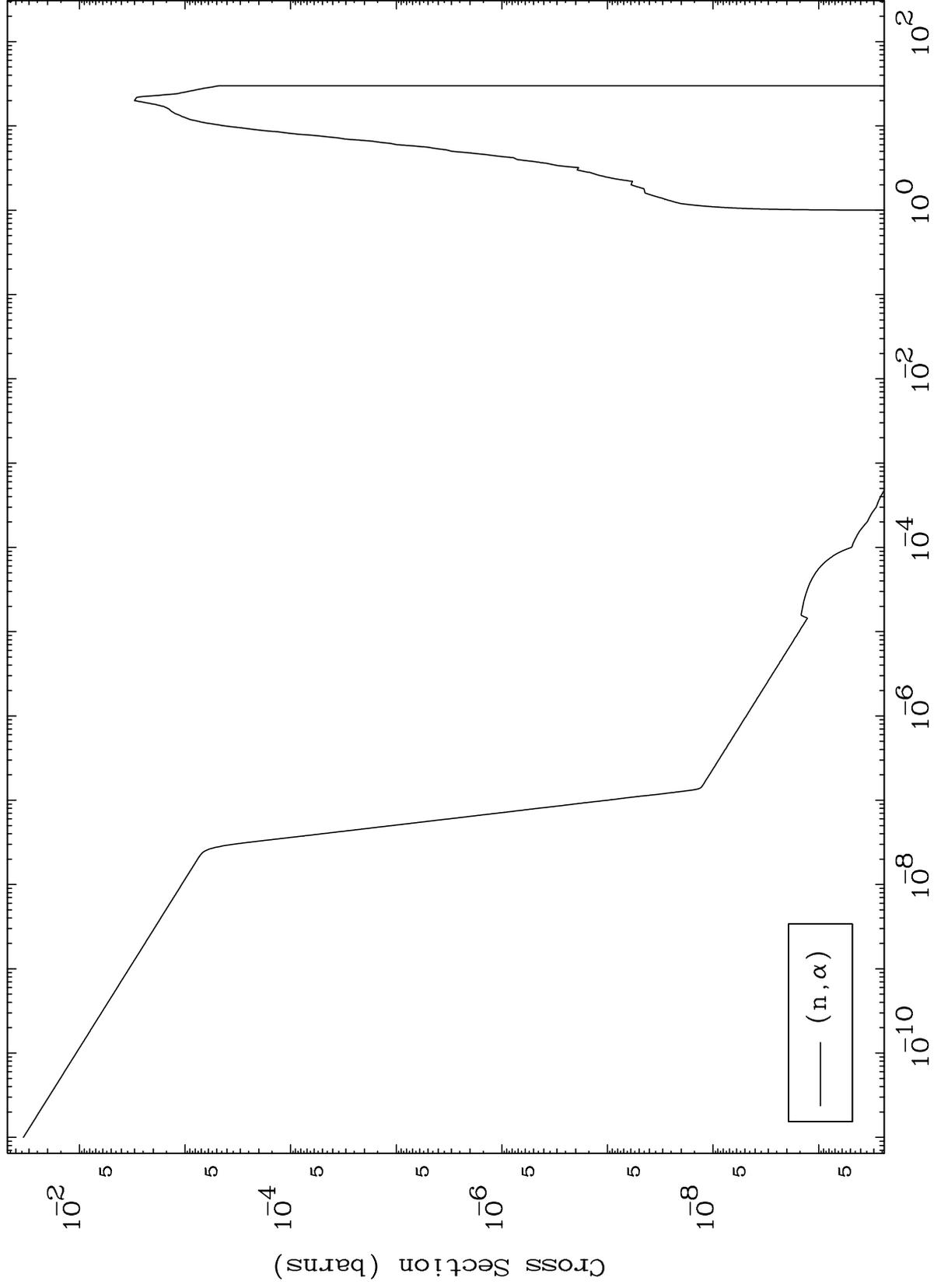
104-Rf-257m



MAT 457

$(n, \alpha)$  Levels  
293 Kelvin Cross Sections

104-Rf-257m



14

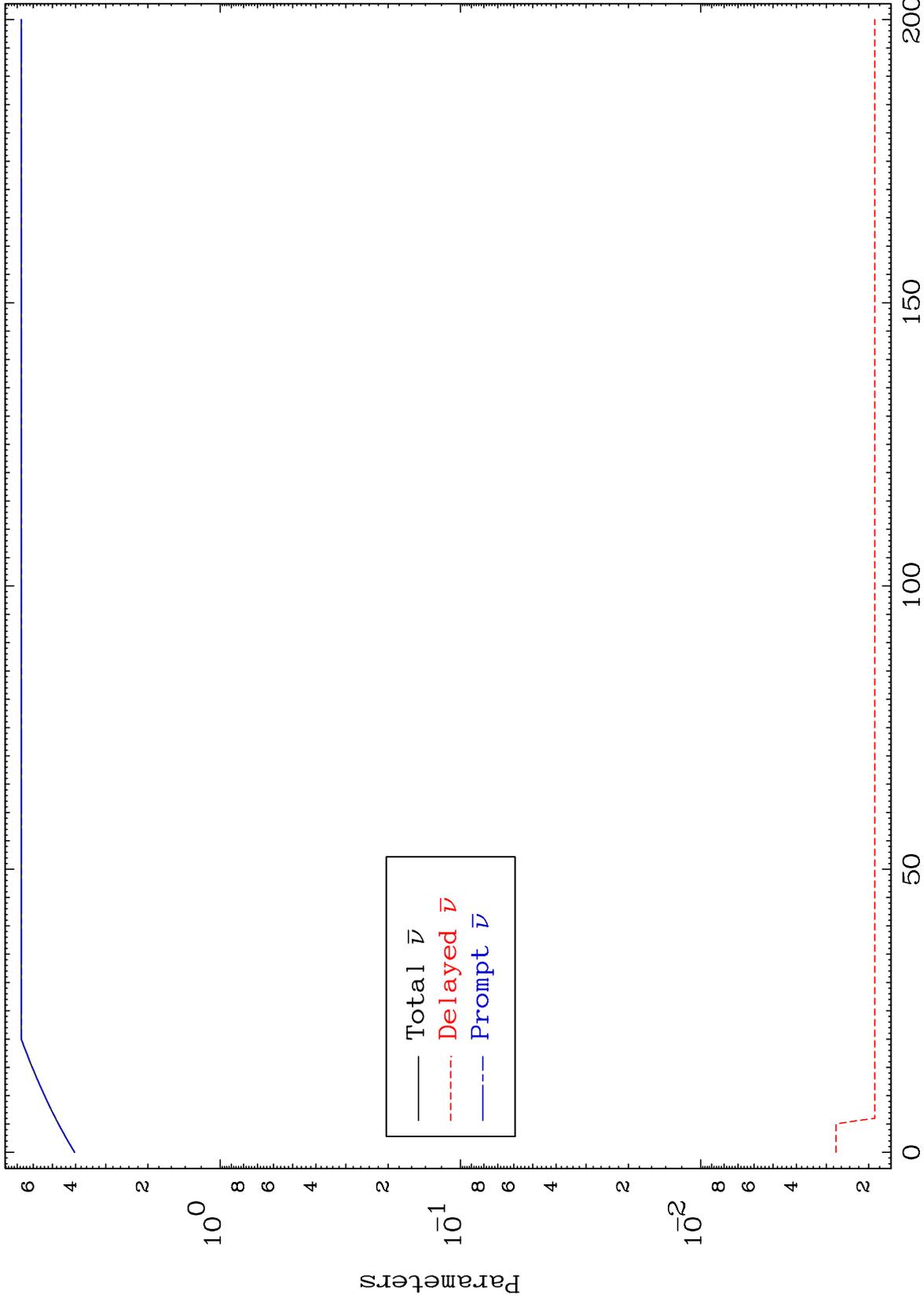
Incident Energy (MeV)

104-Rf-257m

MAT 457

Energy Release  
Parameters

104-Rf-257m

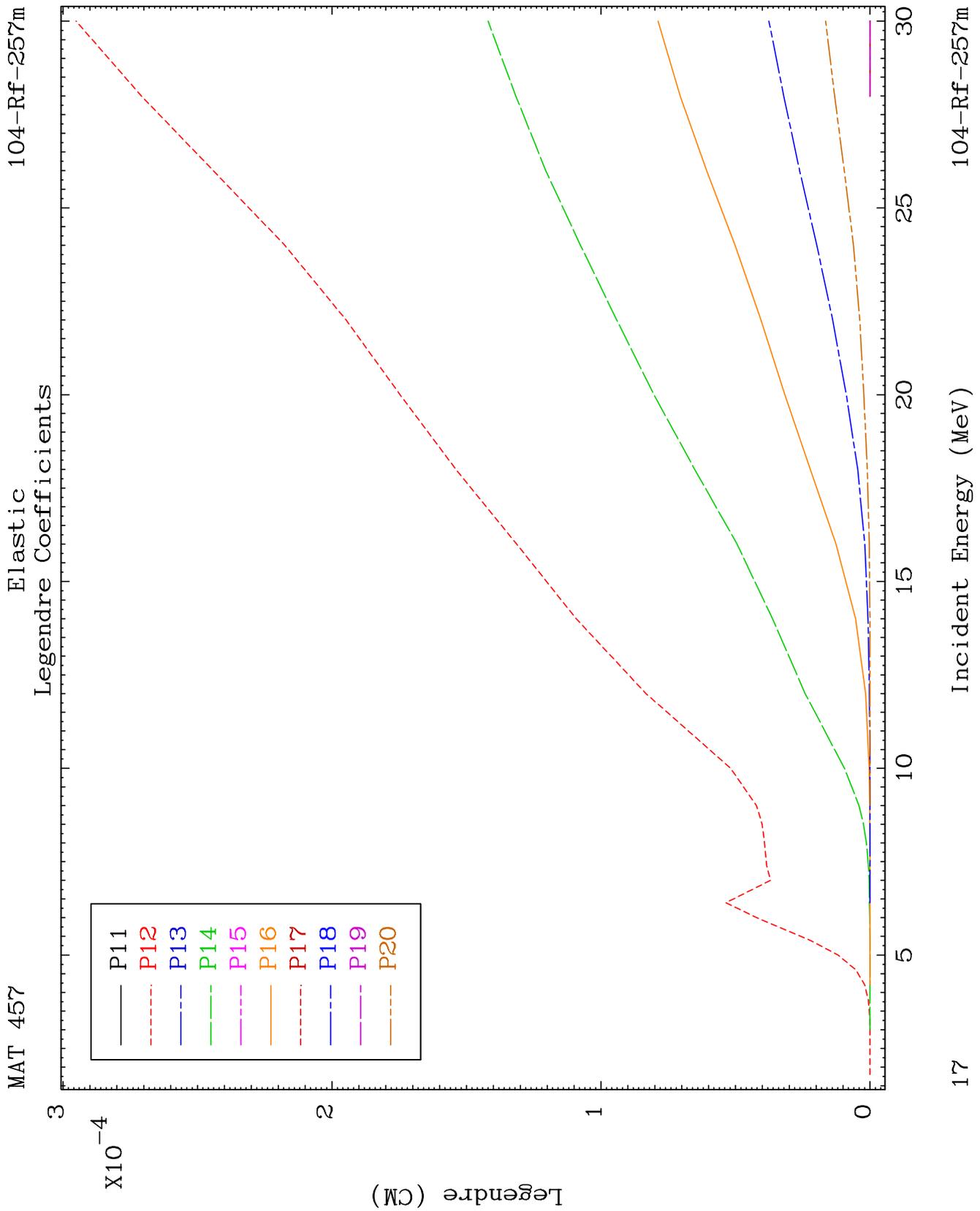


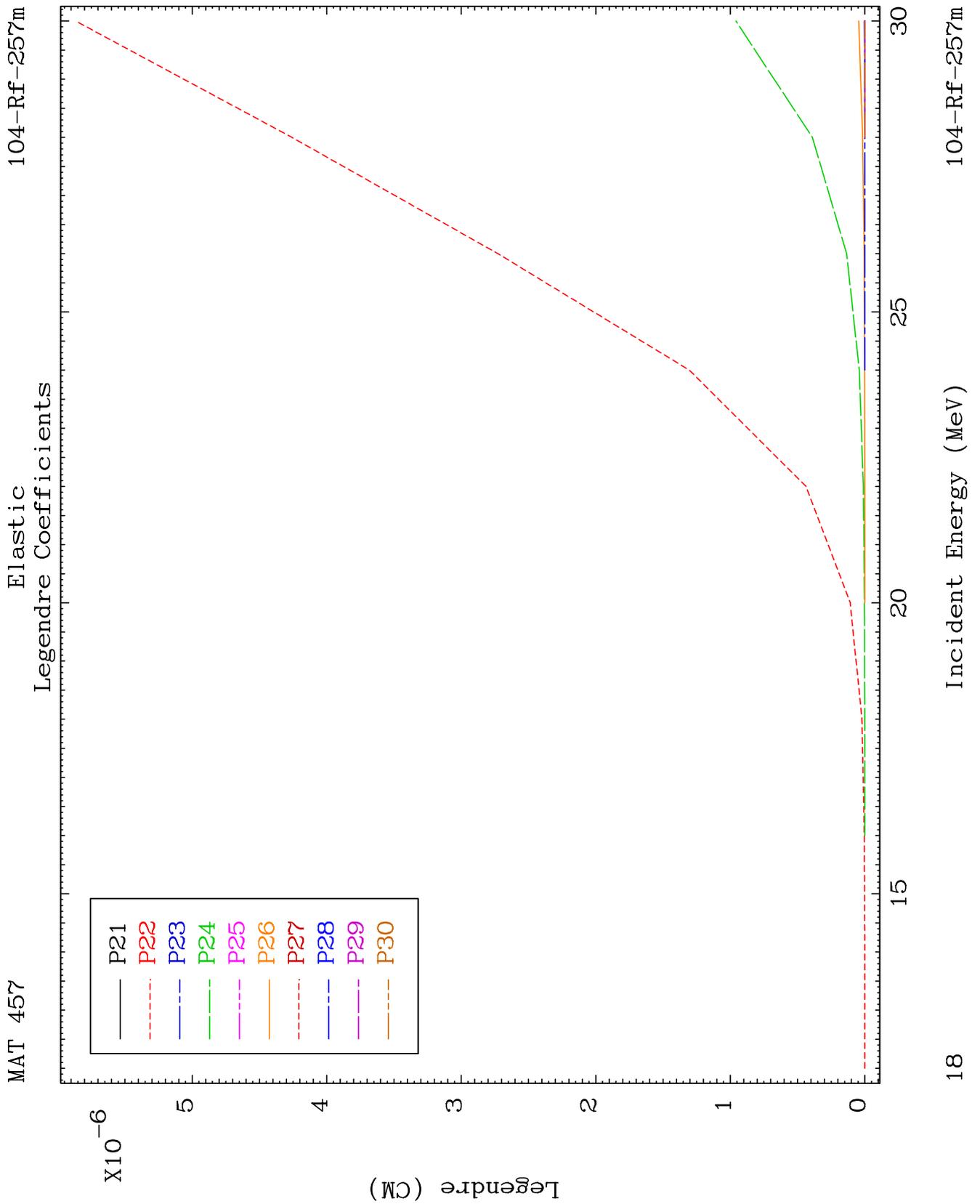
15

Incident Energy (MeV)

104-Rf-257m



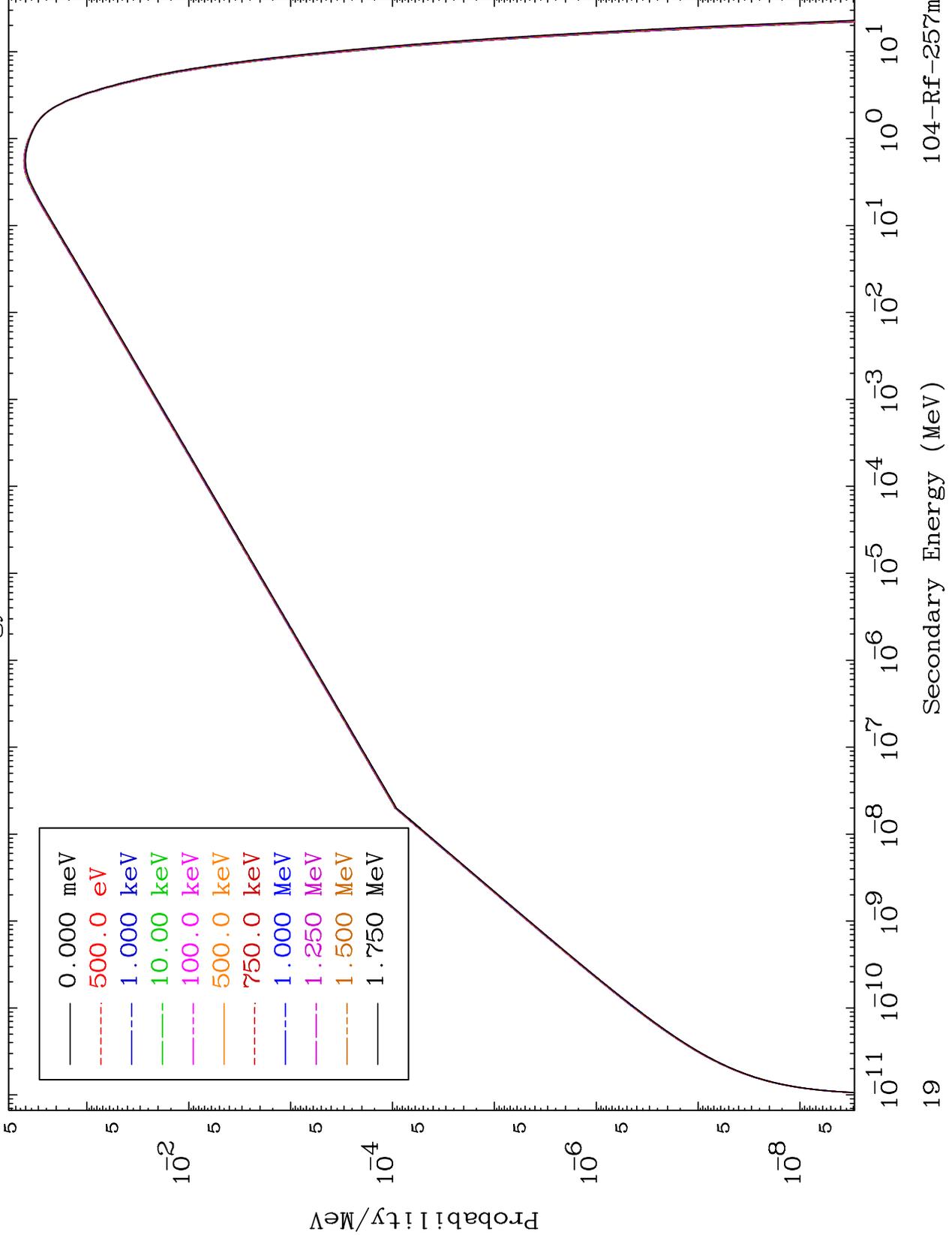




MAT 457

### Fission Energy Distributions

104-Rf-257m



104-Rf-257m

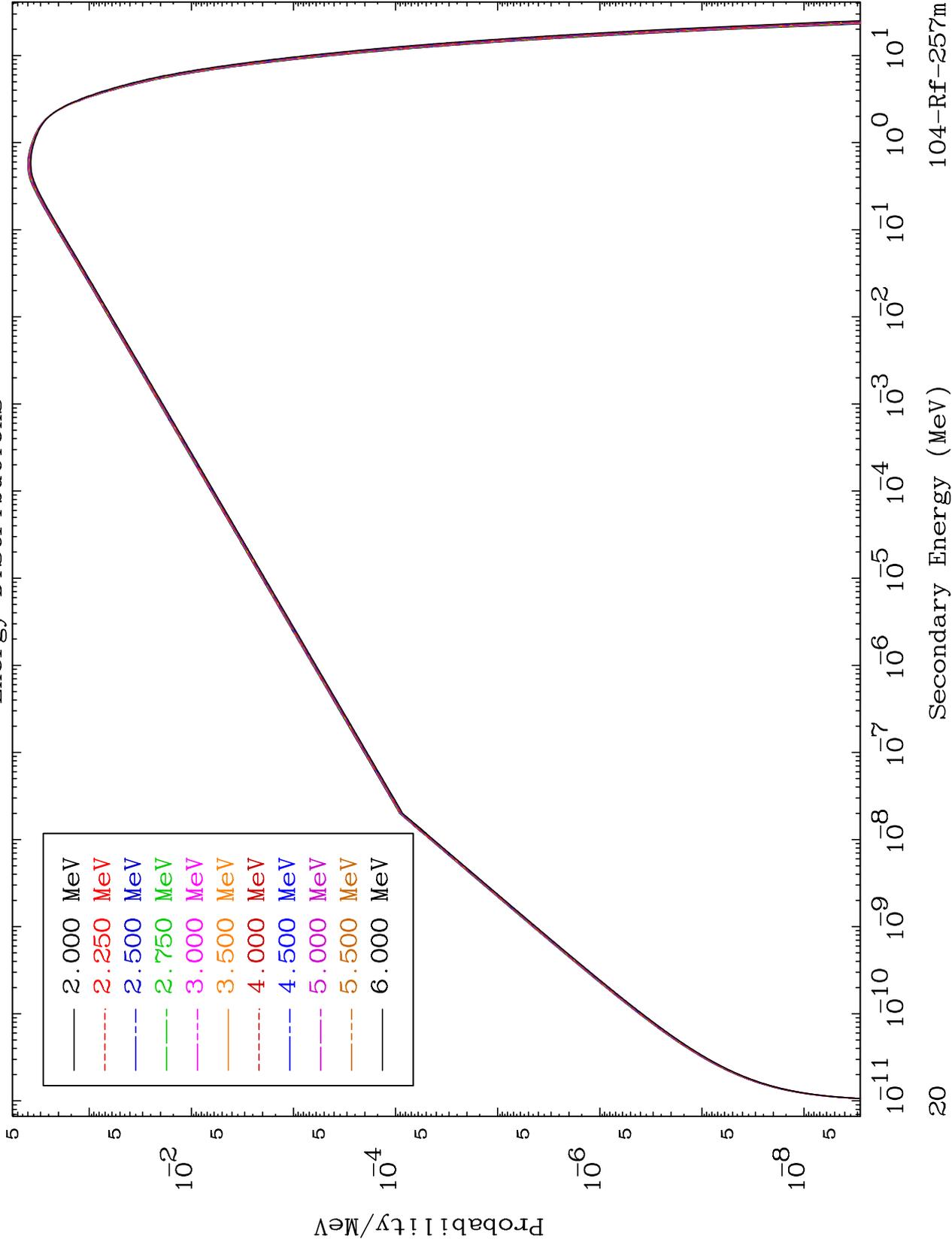
Secondary Energy (MeV)

19

MAT 457

### Fission Energy Distributions

104-Rf-257m



104-Rf-257m

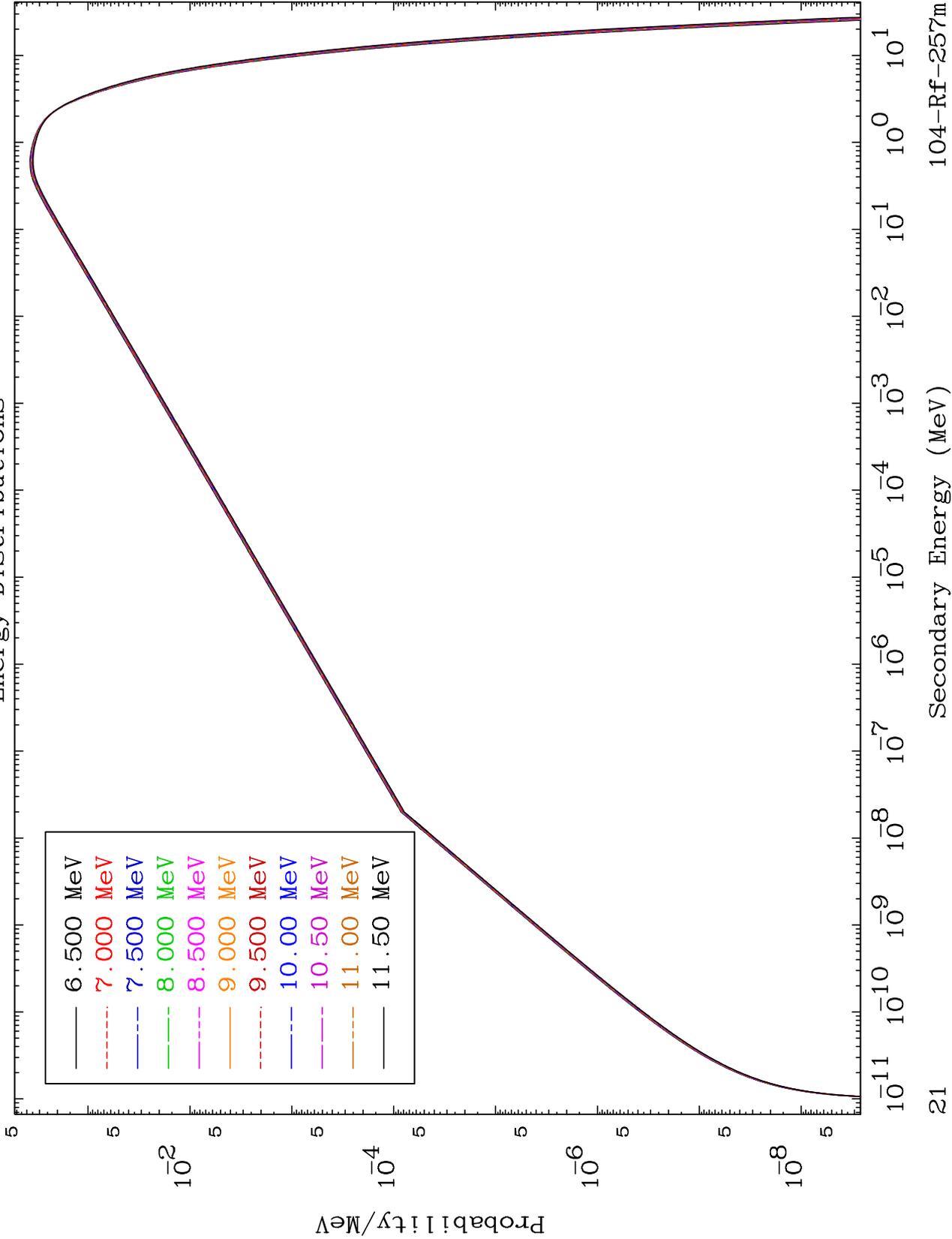
Secondary Energy (MeV)

20

MAT 457

### Fission Energy Distributions

104-Rf-257m

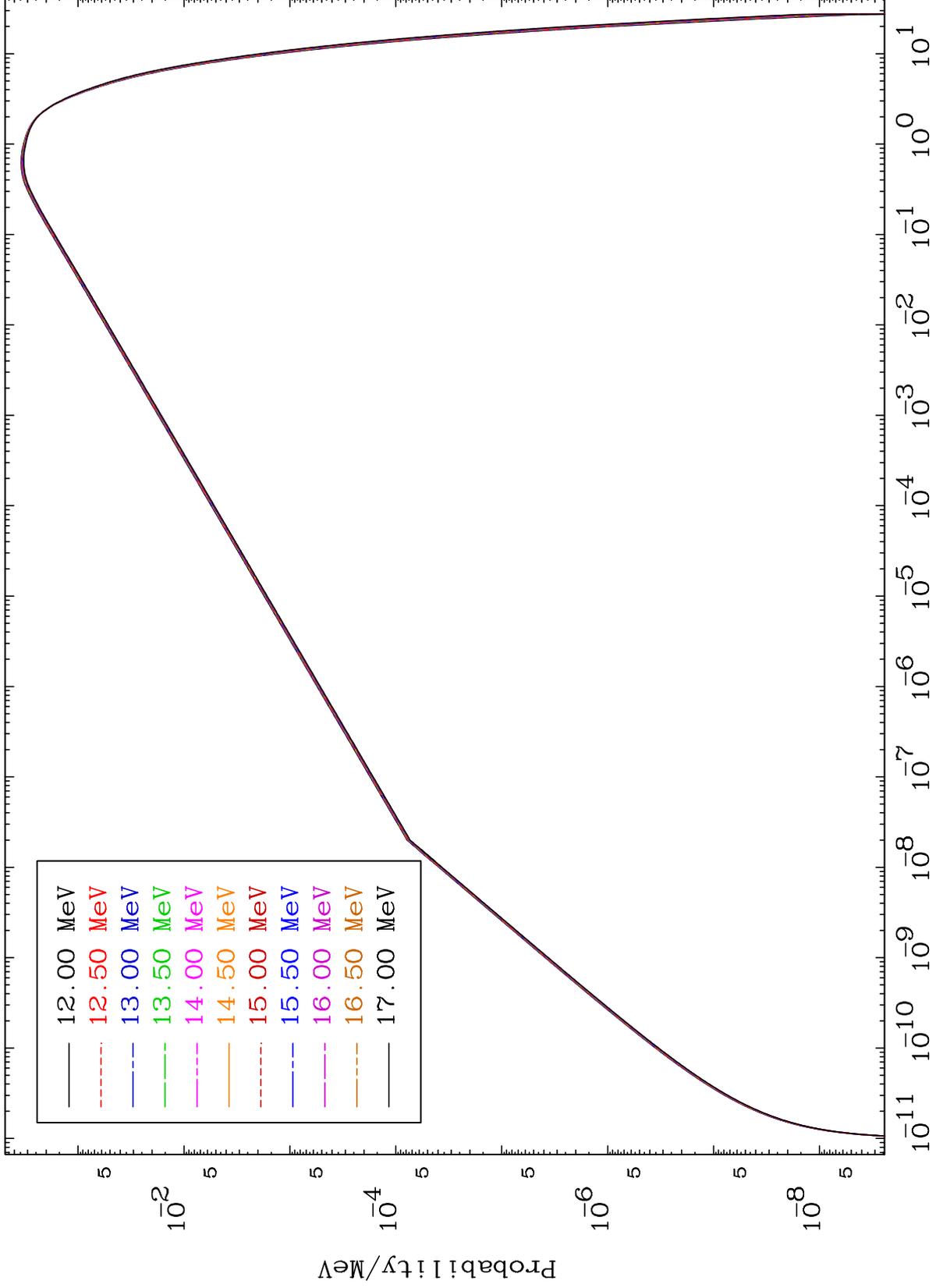


104-Rf-257m

MAT 457

### Fission Energy Distributions

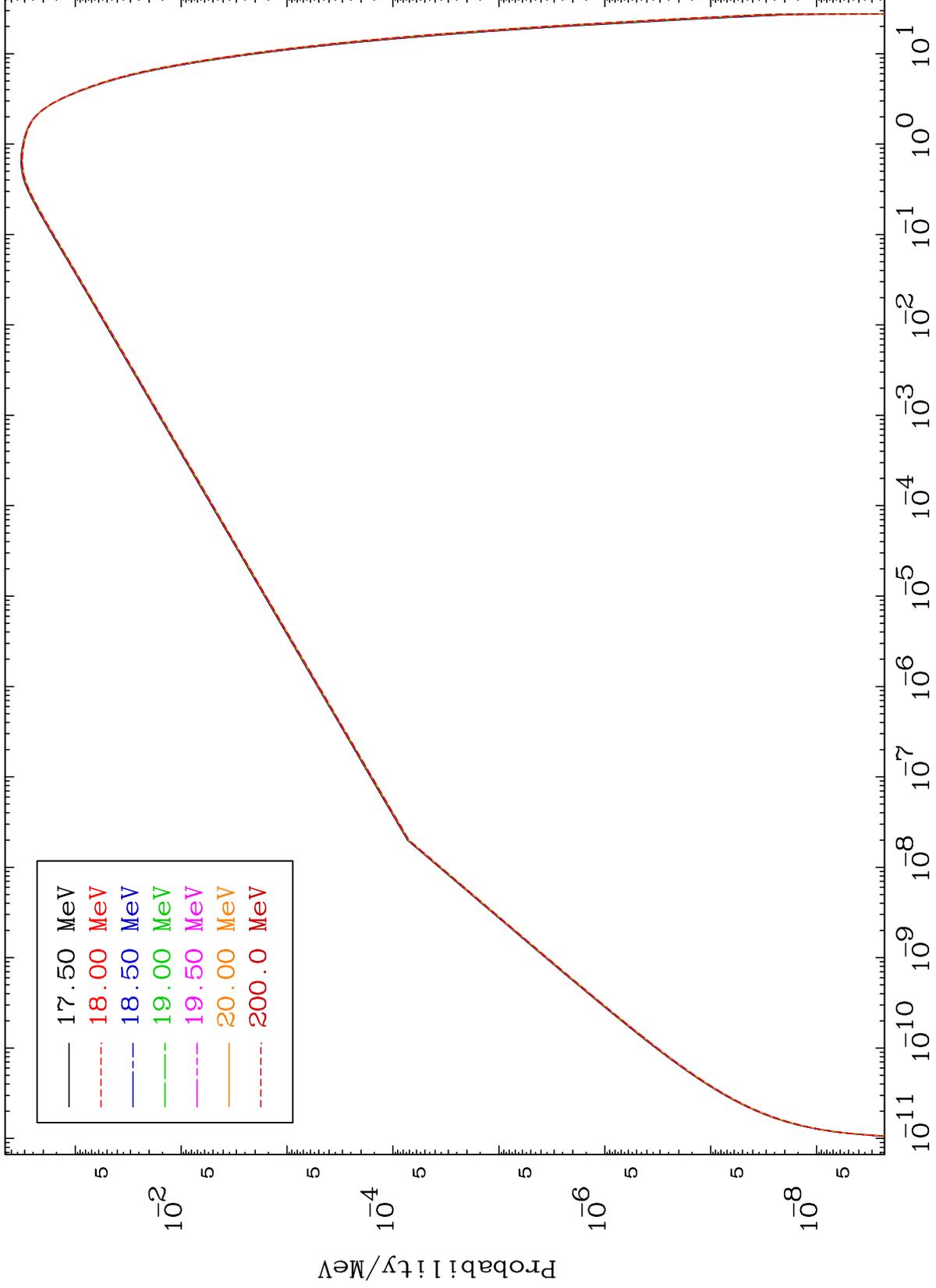
104-Rf-257m



MAT 457

### Fission Energy Distributions

104-Rf-257m



23

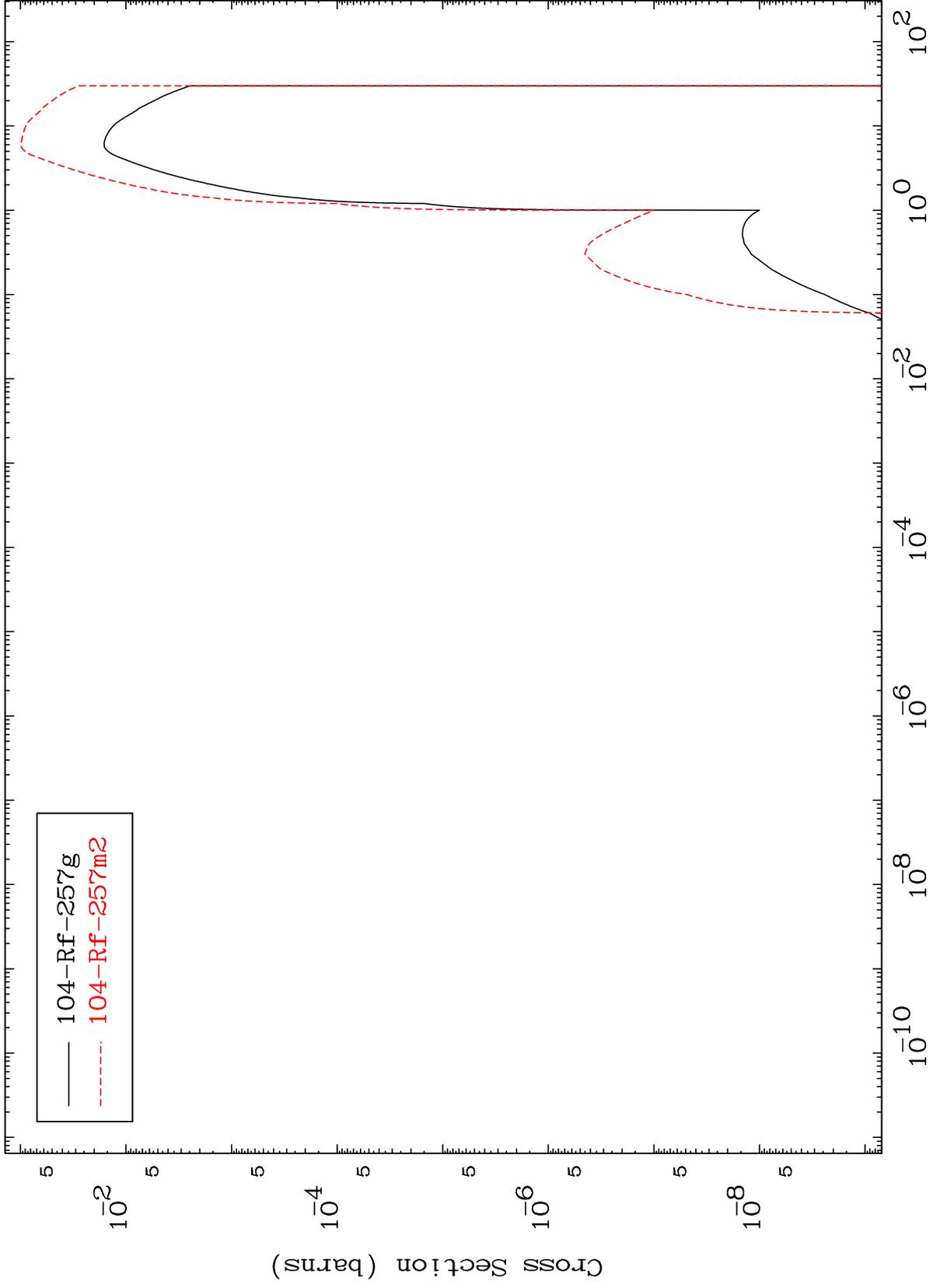
Secondary Energy (MeV)

104-Rf-257m

MAT 457

Inelastic  
Radionuclide Production Cross Section

104-Rf-257m

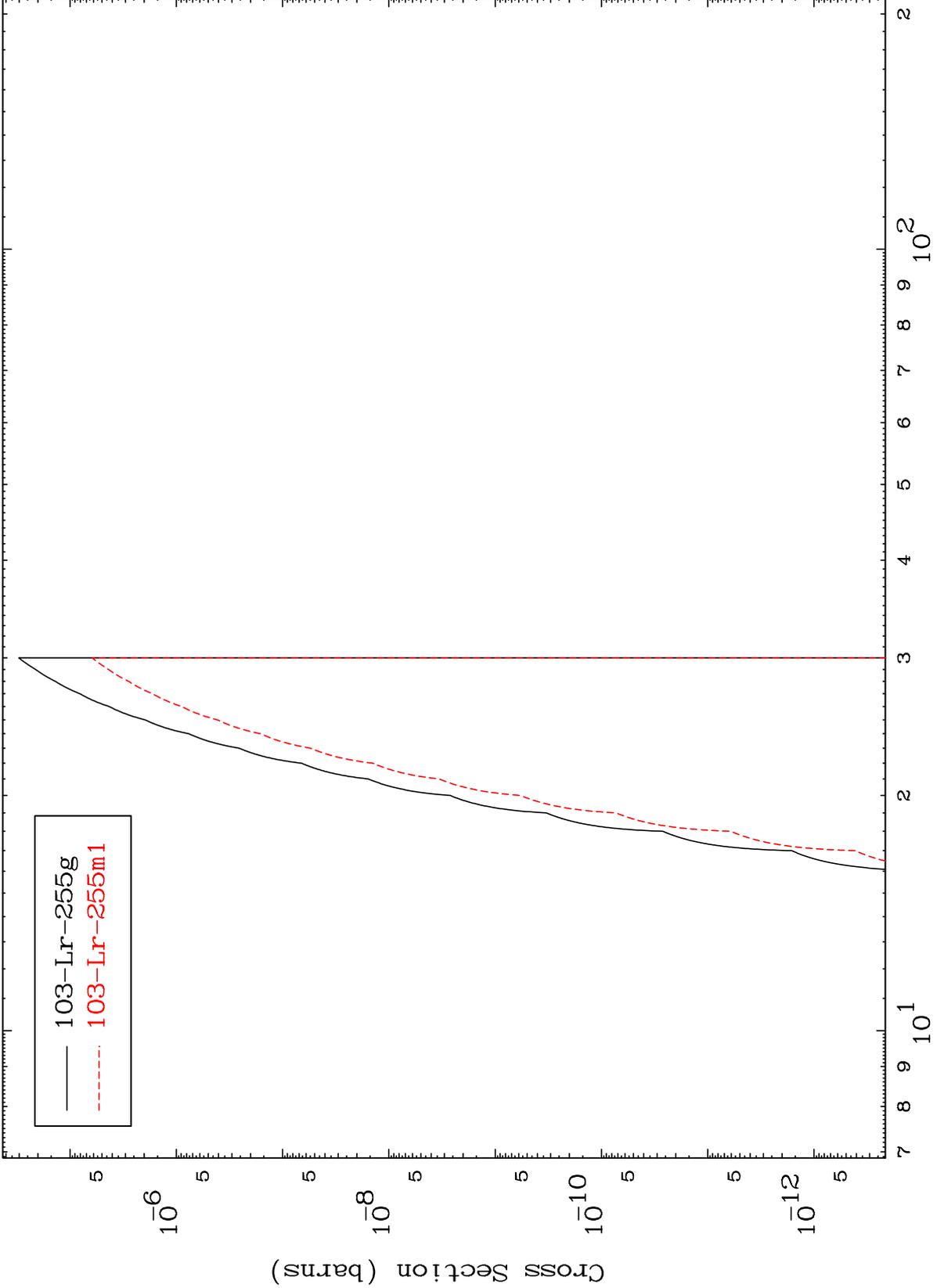


MAT 457

(n,n') d

104-Rf-257m

Radionuclide Production Cross Section



25

Incident Energy (MeV)

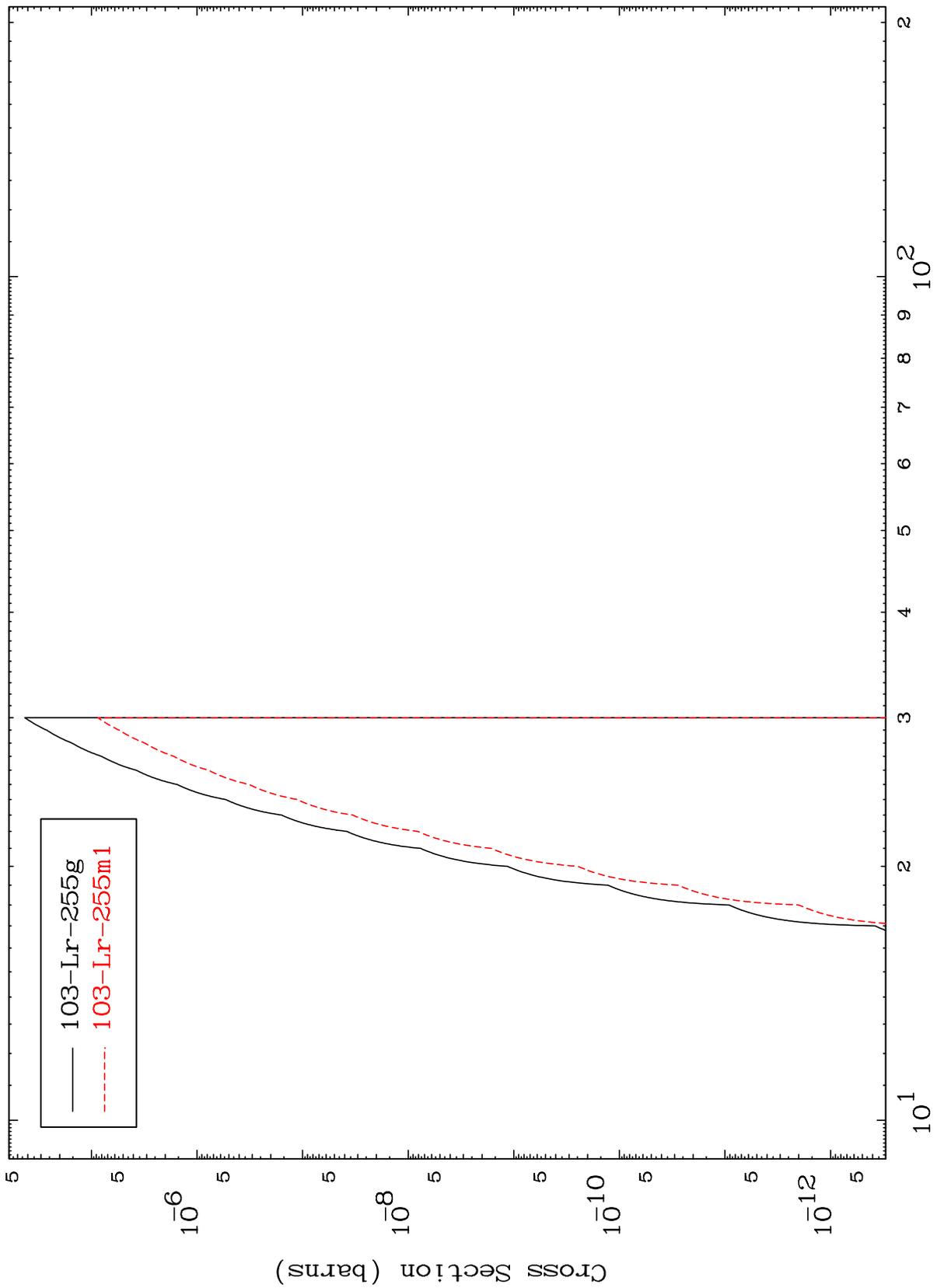
104-Rf-257m

MAT 457

(n,2n) p

104-Rf-257m

Radionuclide Production Cross Section



Incident Energy (MeV)

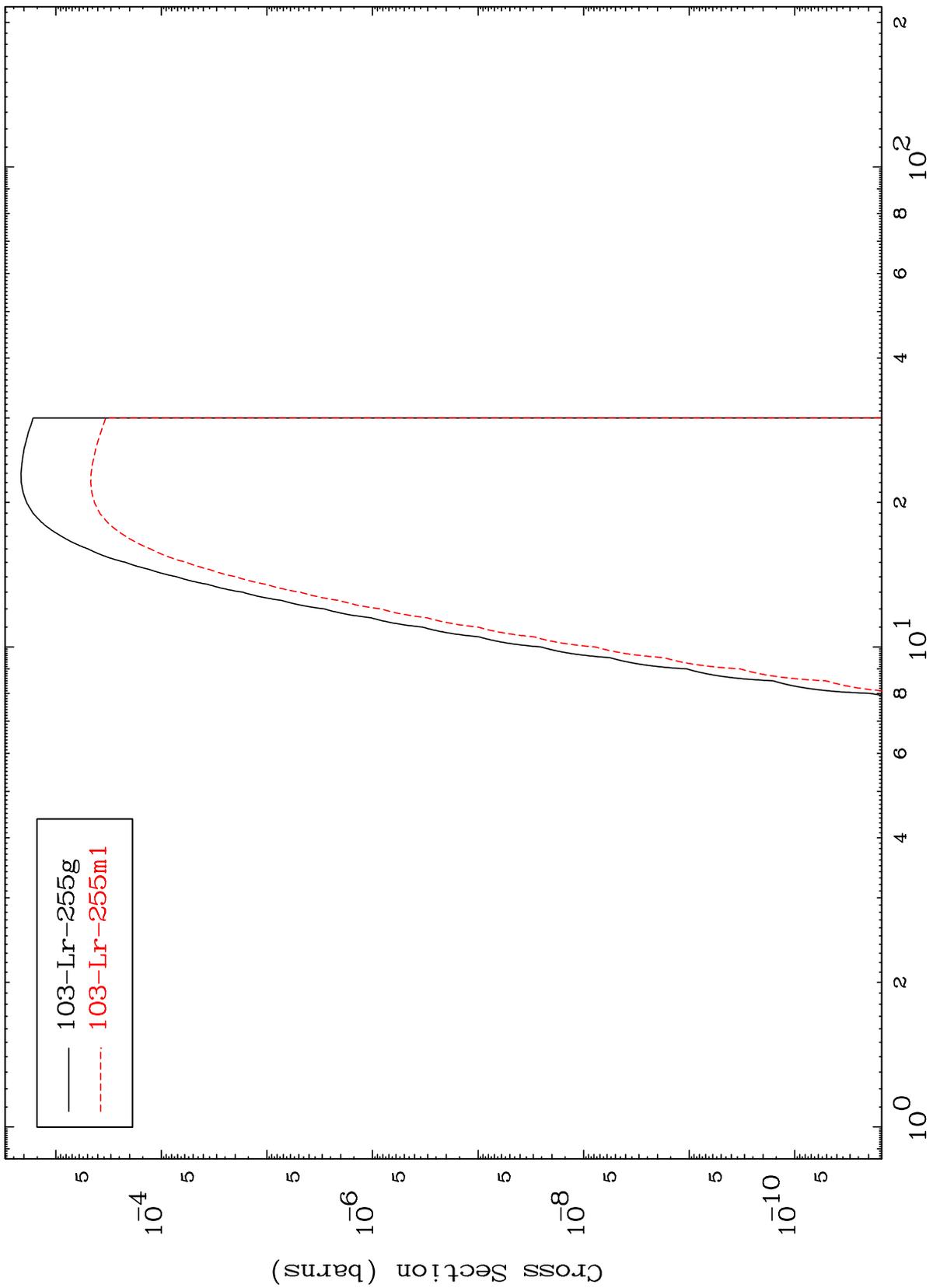
104-Rf-257m

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MAT 457

104-Rf-257m

(n, t)  
Radionuclide Production Cross Section



104-Rf-257m

Incident Energy (MeV)

27