

Program EVALPLOT
(Version 2021-1)

by

Dermott E. Cullen
(Present Contact Information)

Dermott E. Cullen
1466 Hudson Way
Livermore, CA 94550
U.S.A.

Tele: 925-443-1911

E.Mail:redcullen1@comcast.net

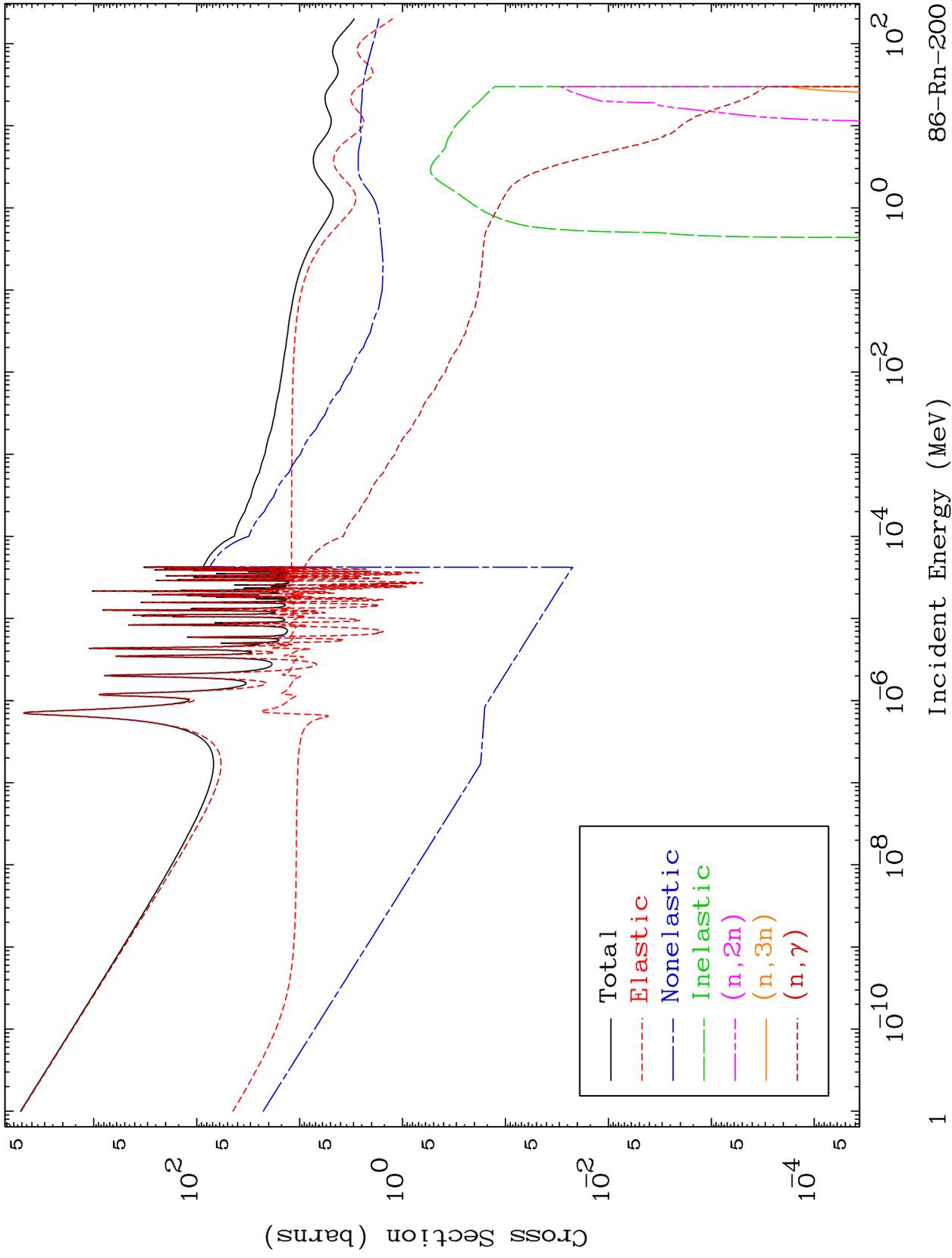
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Press Mouse Button to Start

MAT 8592

Neutron Major
293 Kelvin Cross Sections

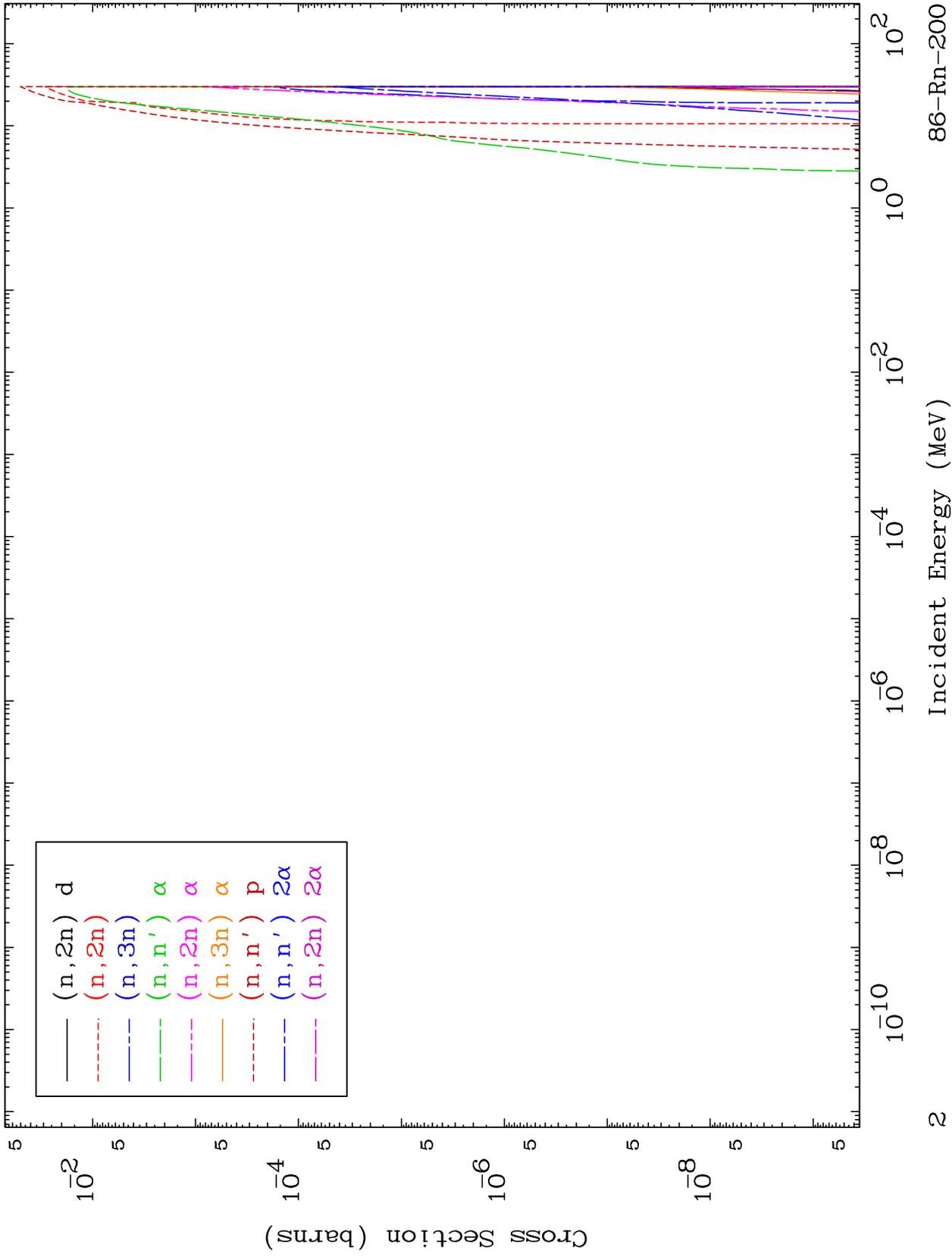
86-Rn-200



MAT 8592

Neutron Absorption
293 Kelvin Cross Sections

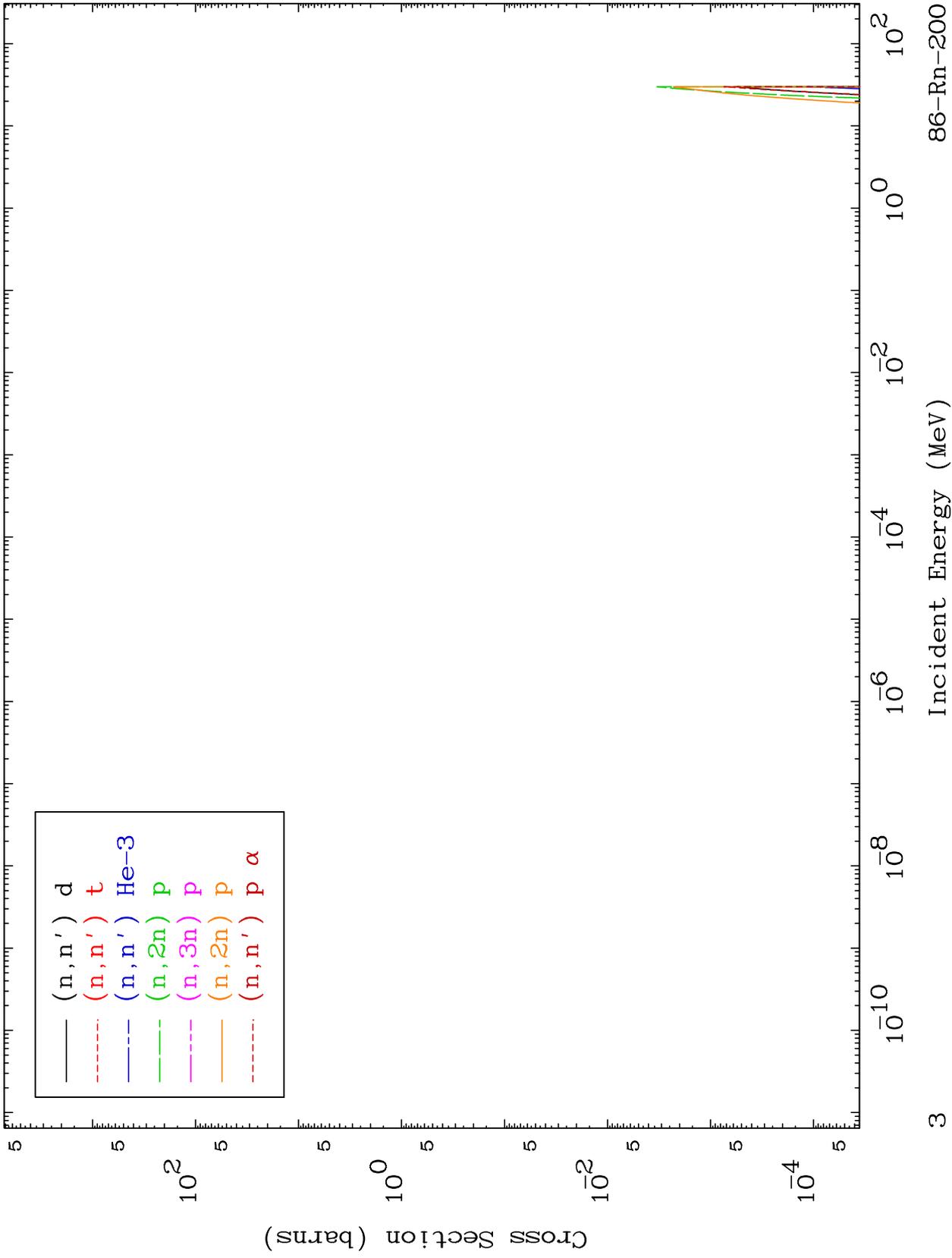
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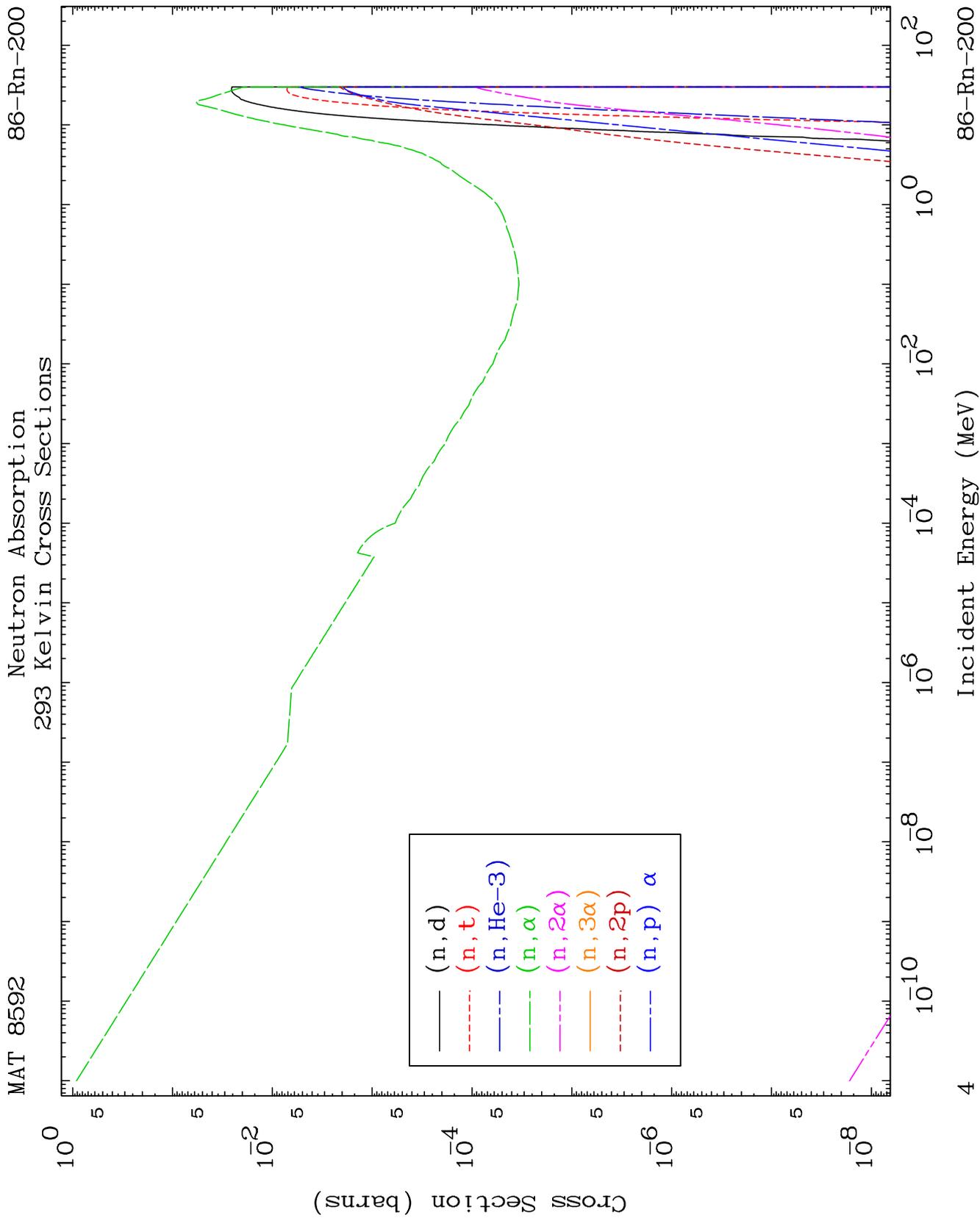


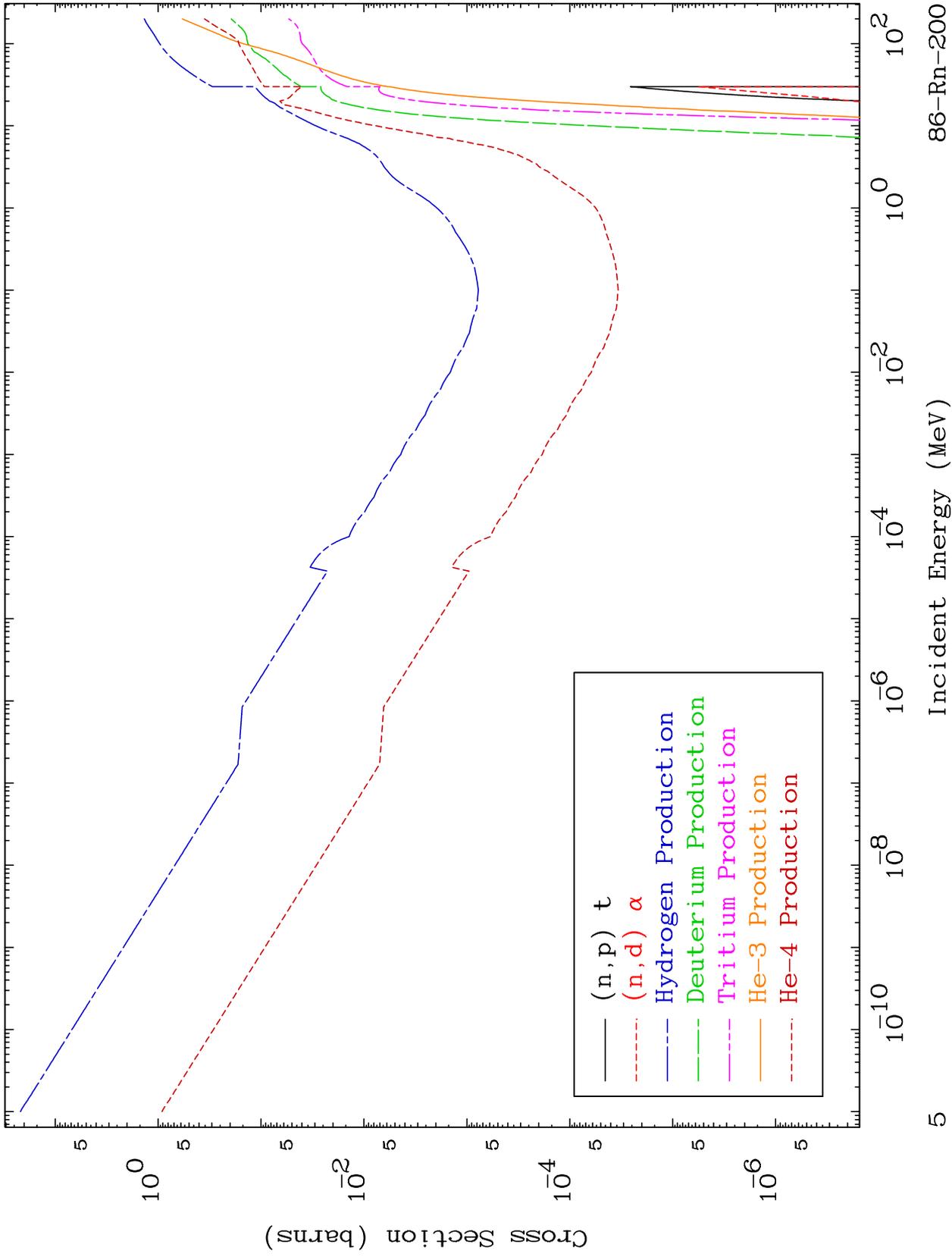
MAT 8592

Neutron Absorption
293 Kelvin Cross Sections

86-Rn-200



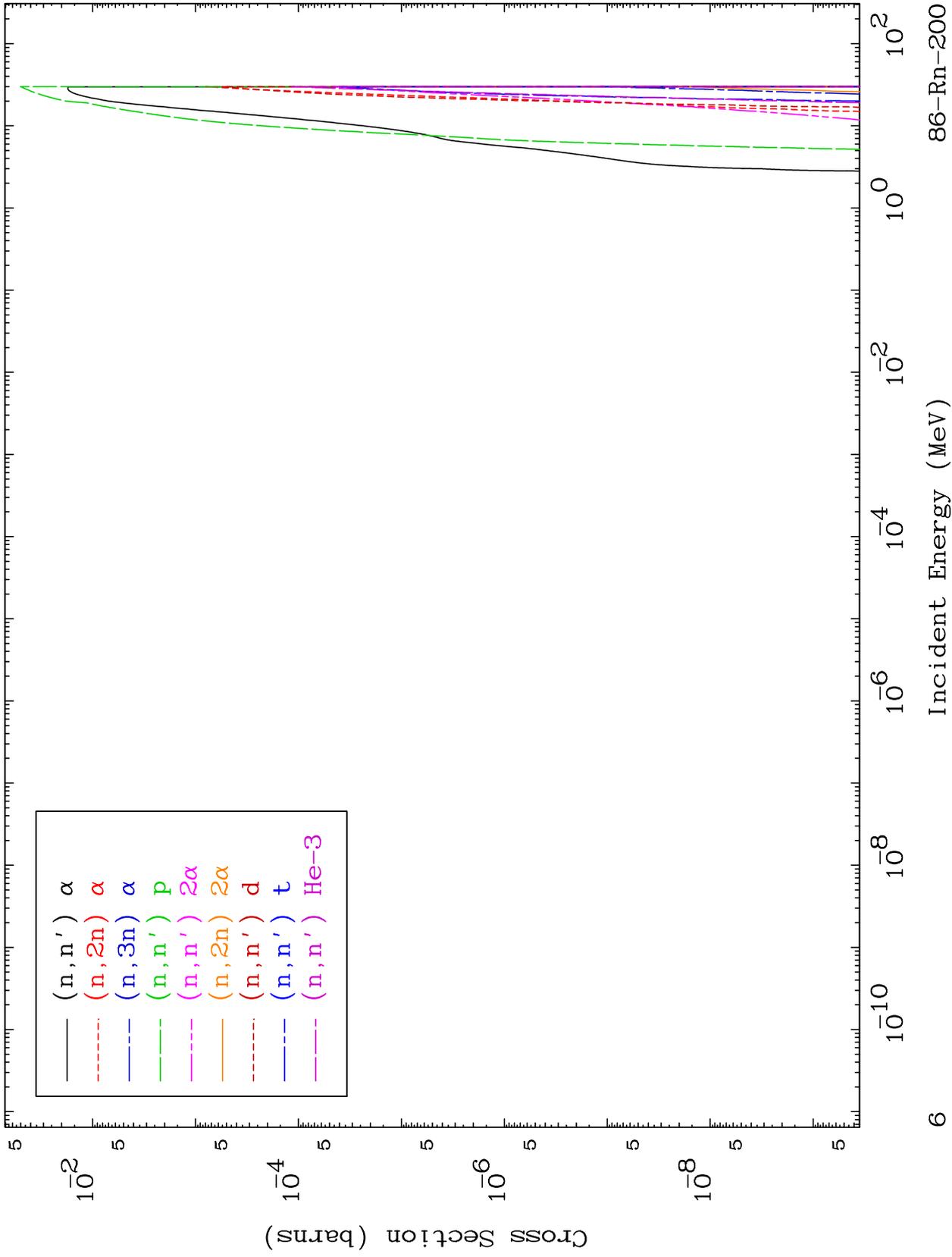




MAT 8592

Charged Particle
293 Kelvin Cross Sections

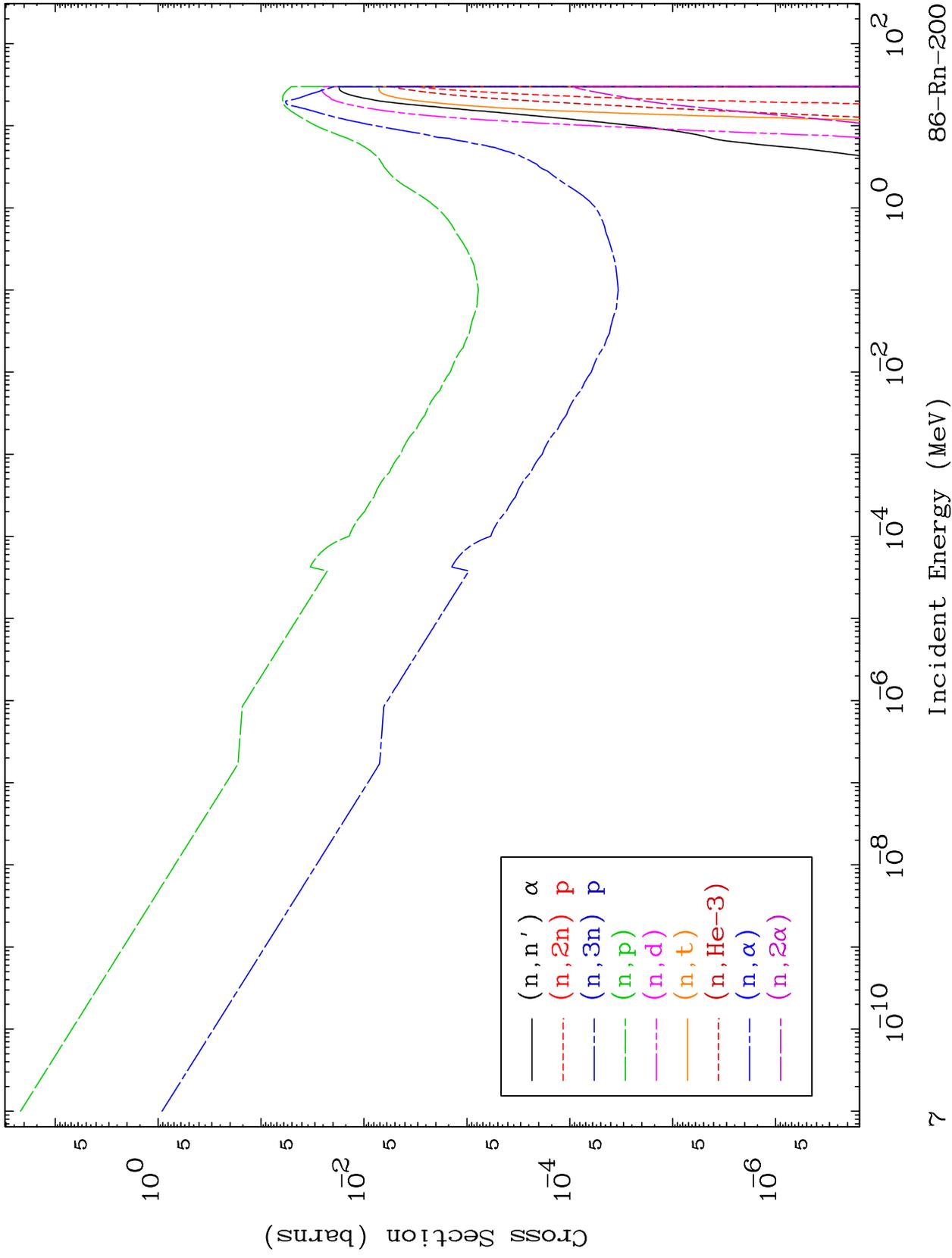
86-Rn-200



MAT 8592

Charged Particle
293 Kelvin Cross Sections

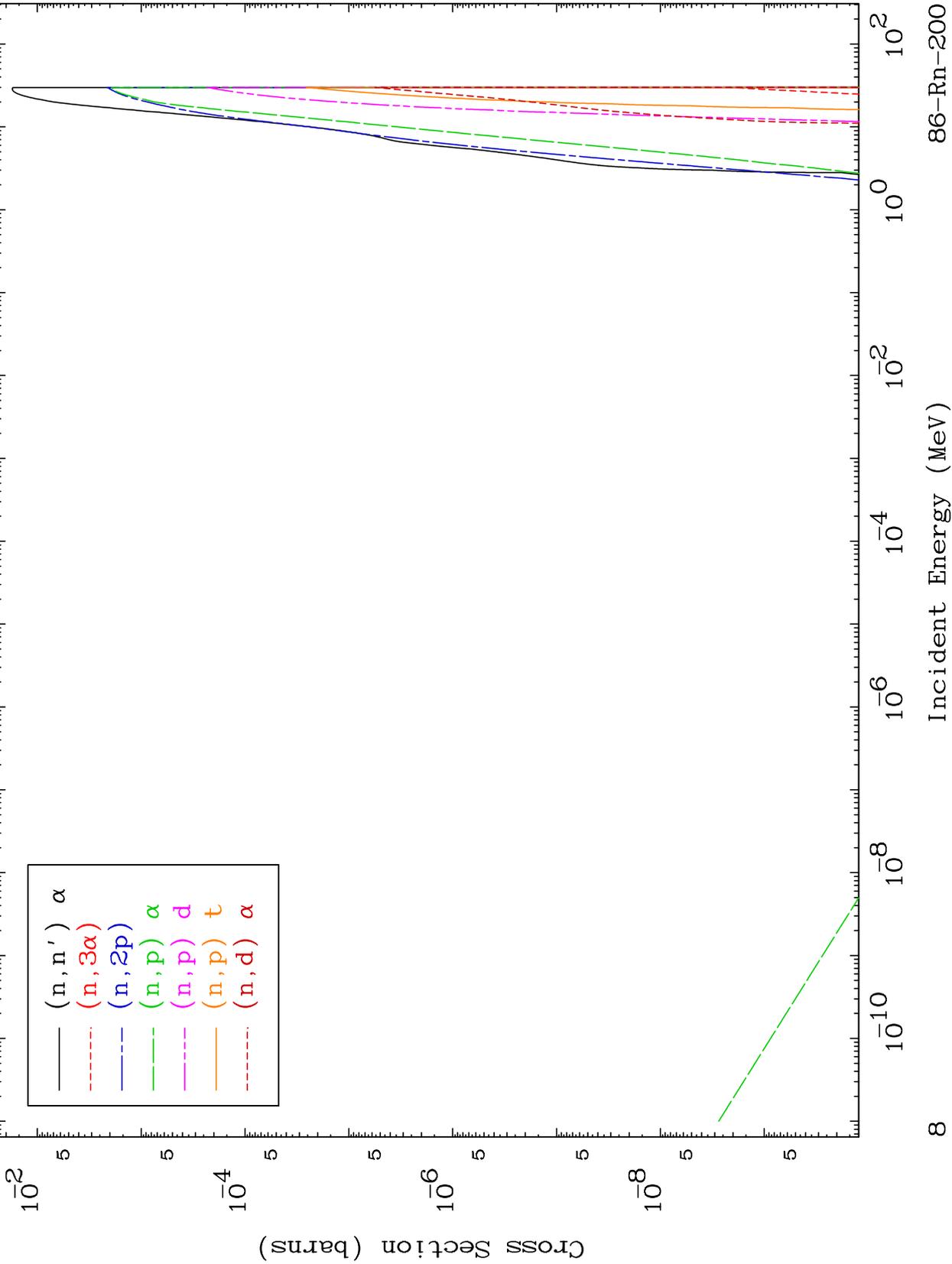
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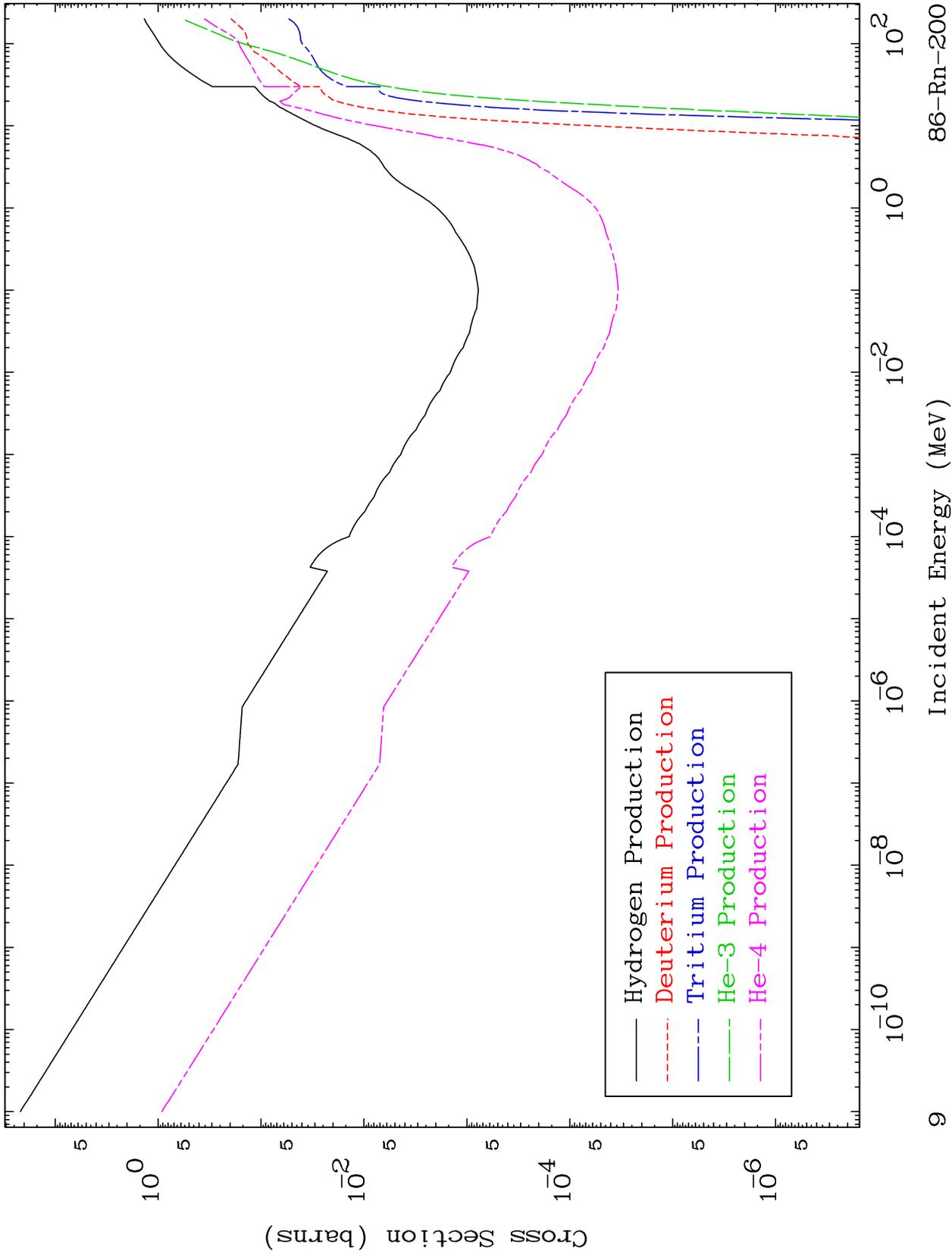


MAT 8592

Charged Particle
293 Kelvin Cross Sections

86-Rn-200

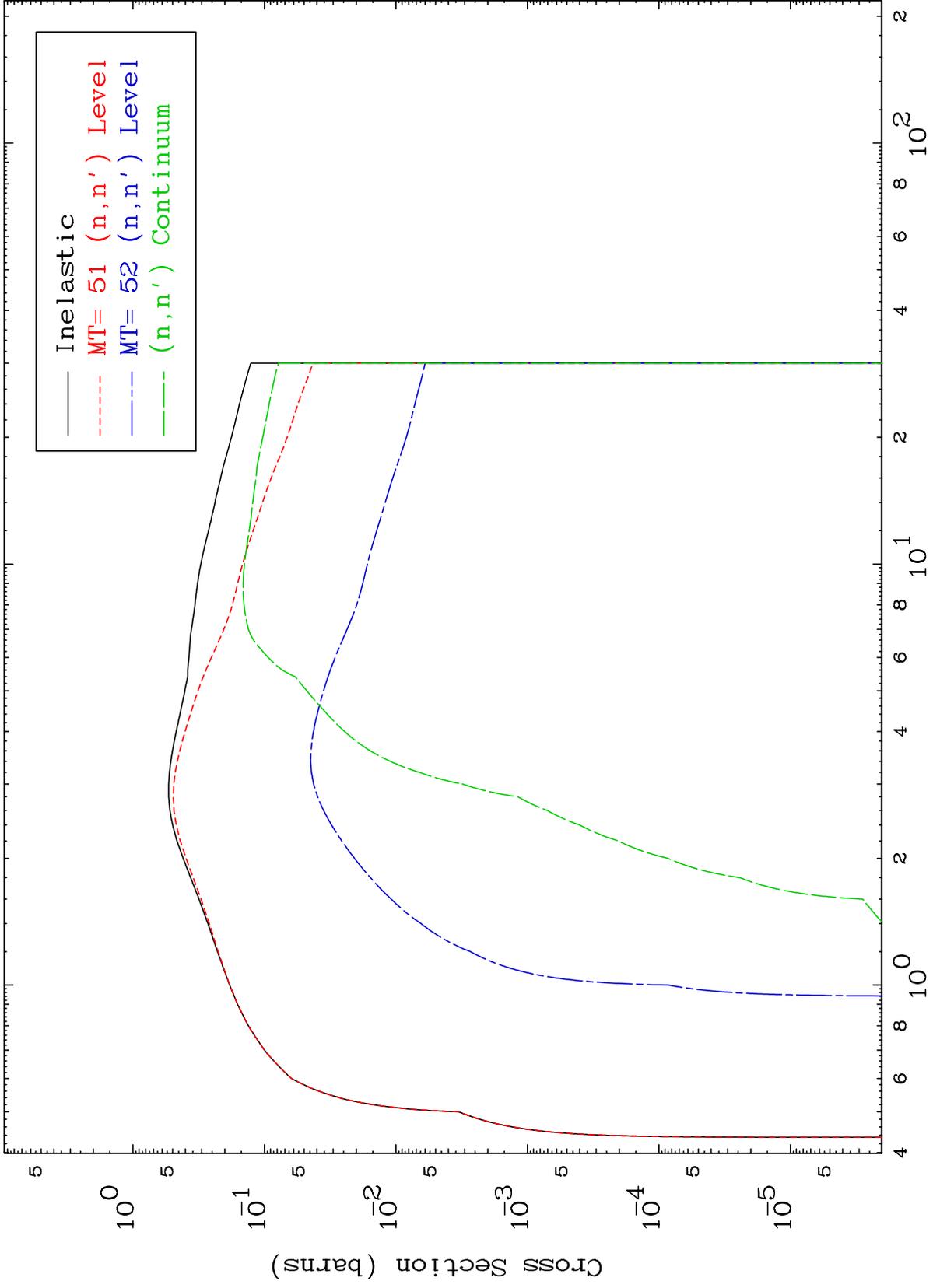




MAT 8592

(n,n') Levels
293 Kelvin Cross Sections

86-Rn-200



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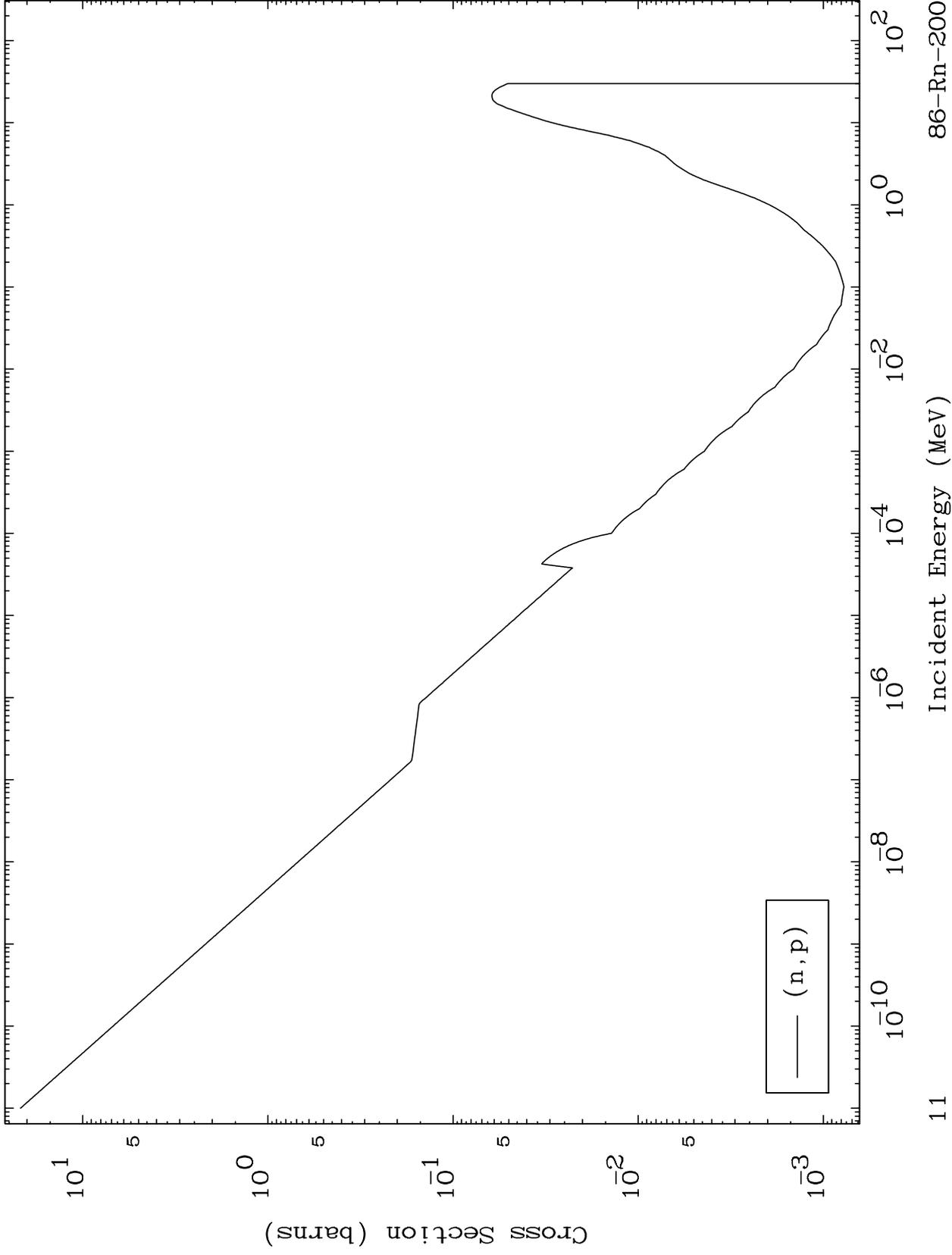
Incident Energy (MeV)

86-Rn-200

MAT 8592

(n,p) Levels
293 Kelvin Cross Sections

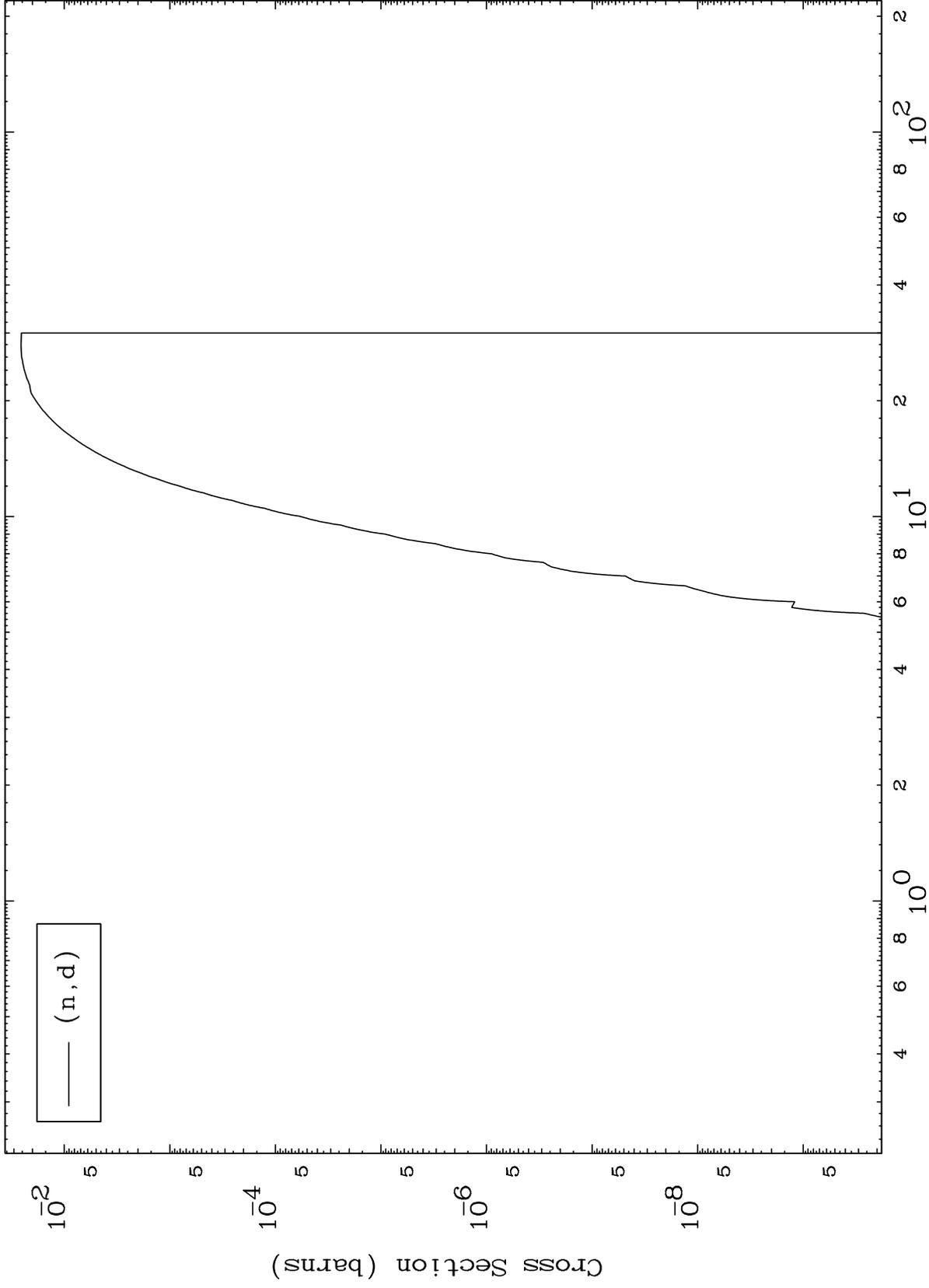
86-Rn-200



MAT 8592

(n,d) Levels
293 Kelvin Cross Sections

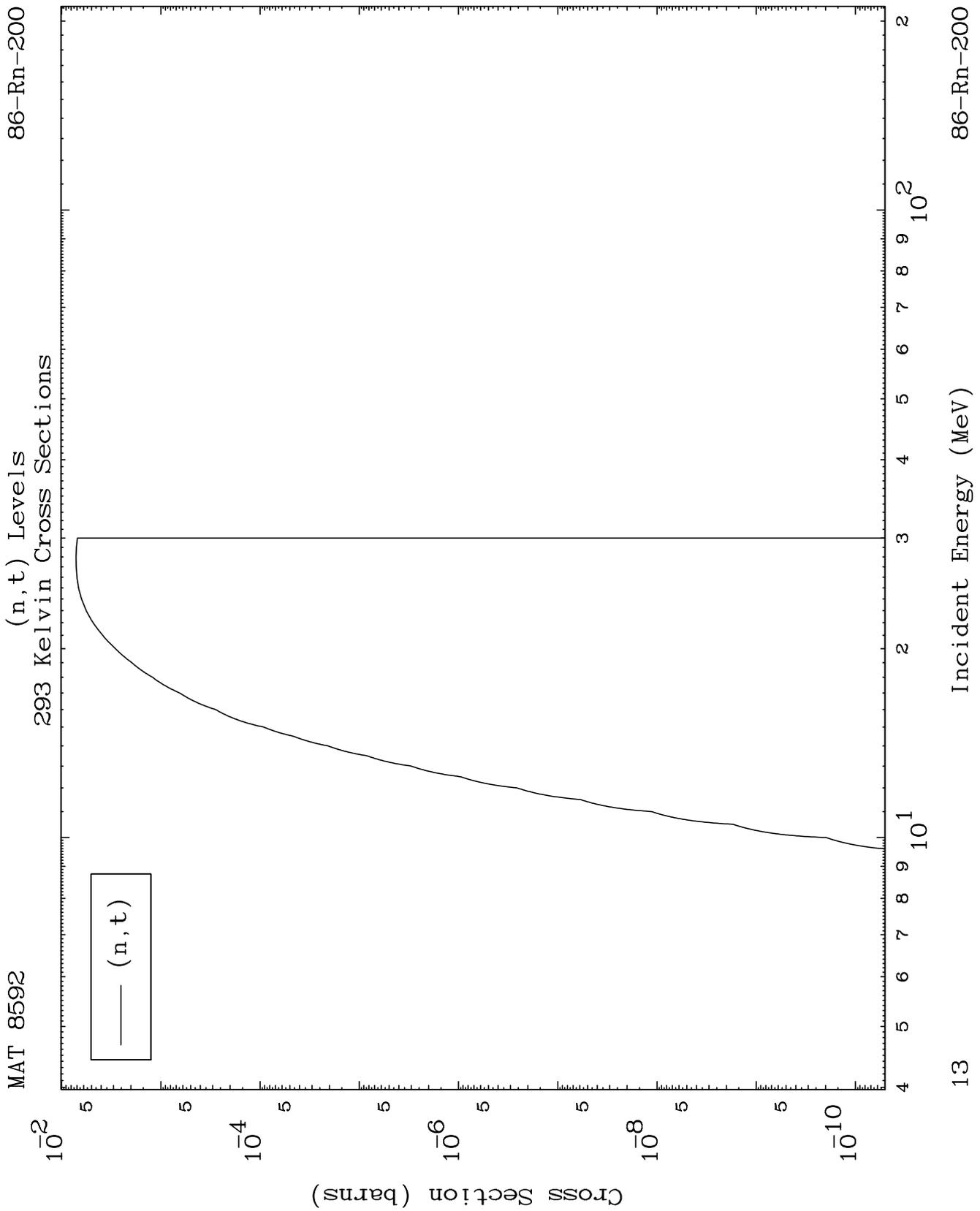
86-Rn-200



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Incident Energy (MeV)

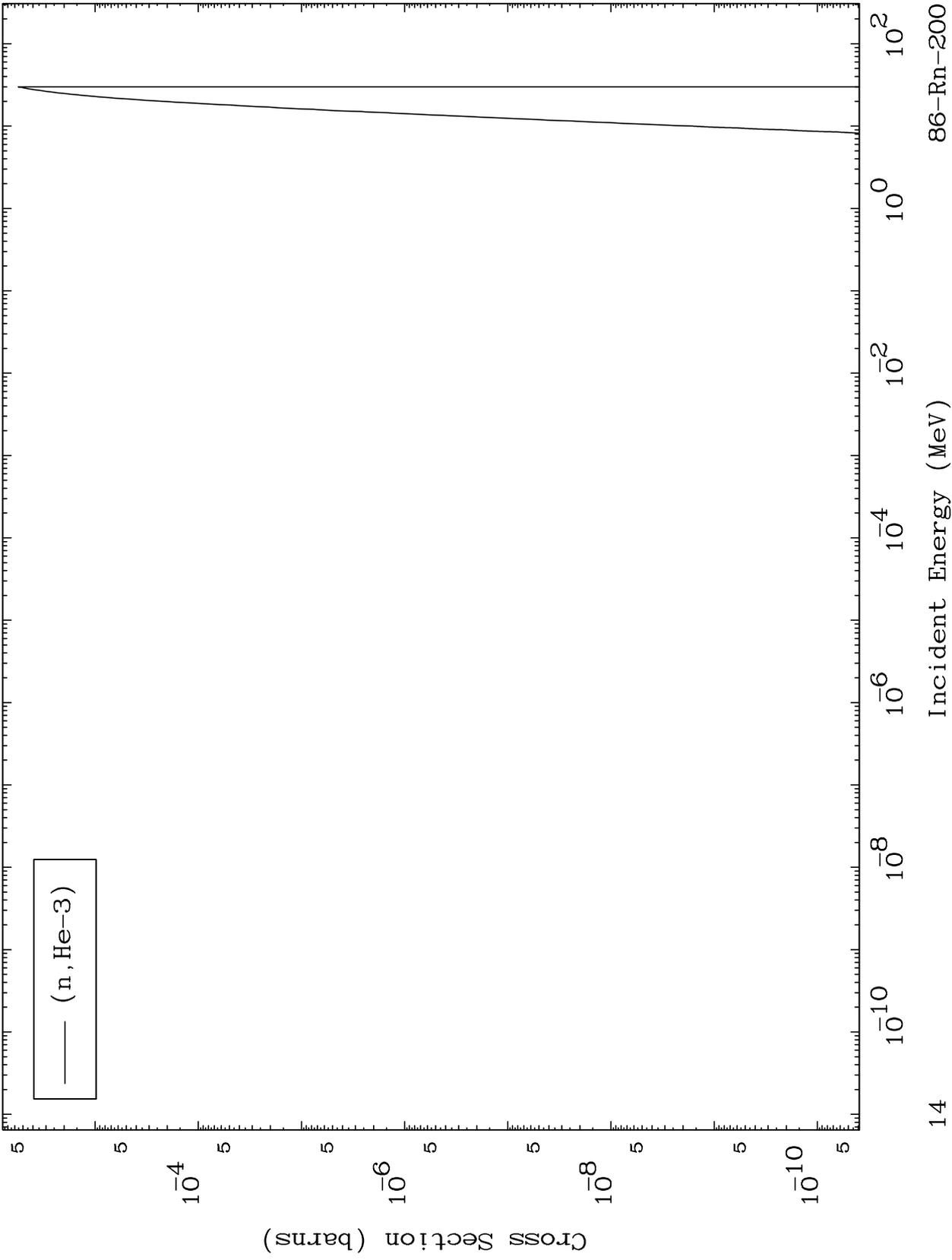
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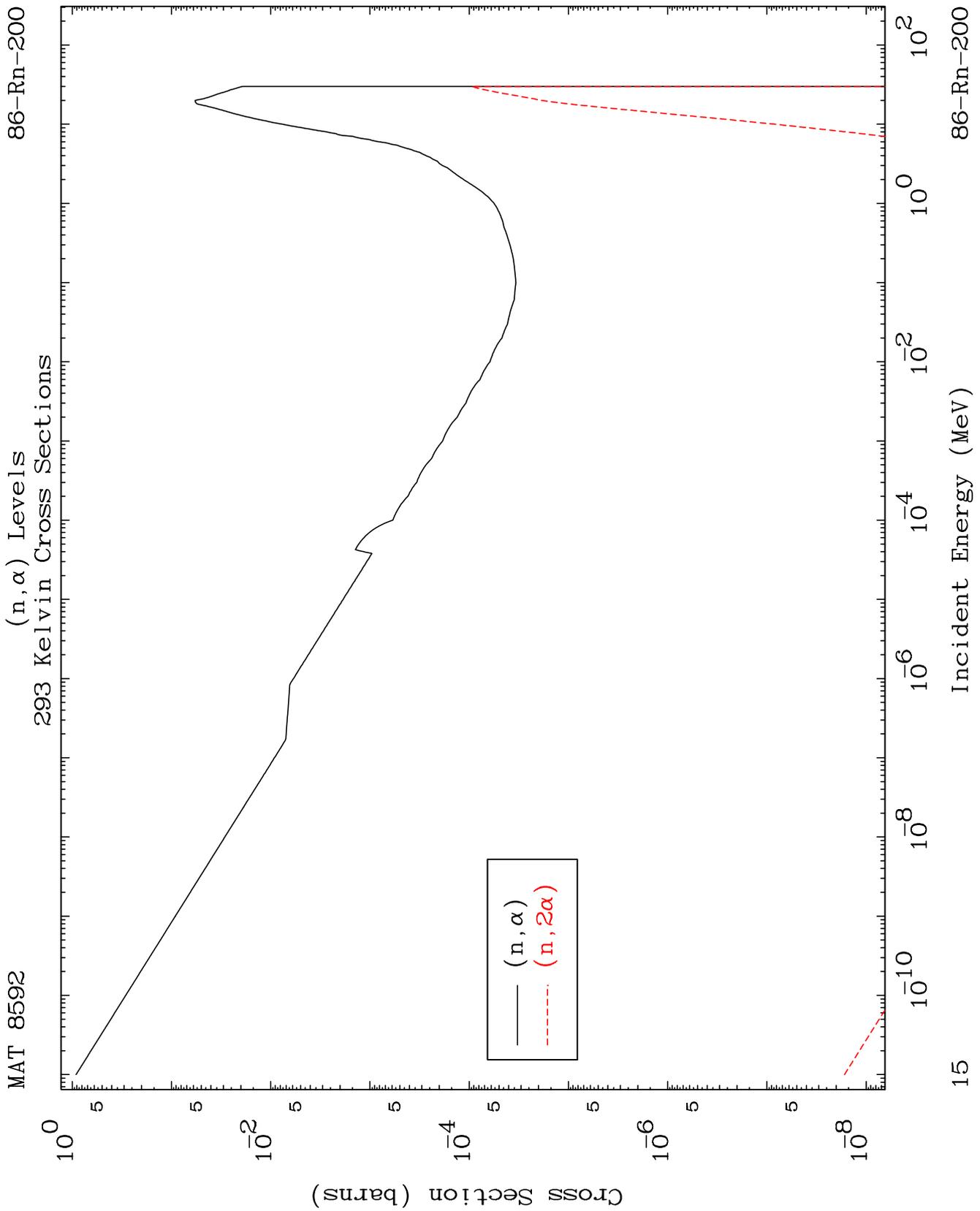


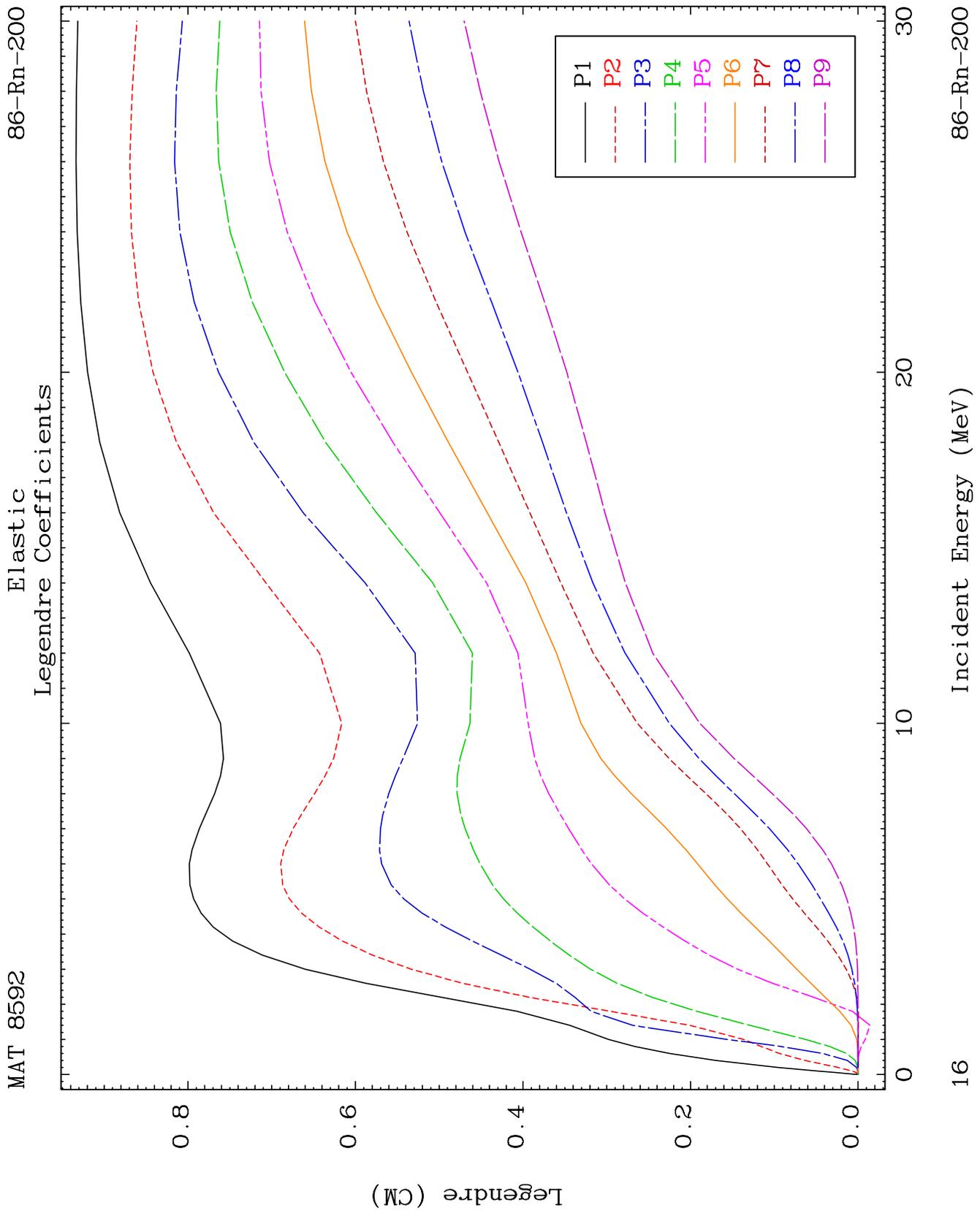
MAT 8592

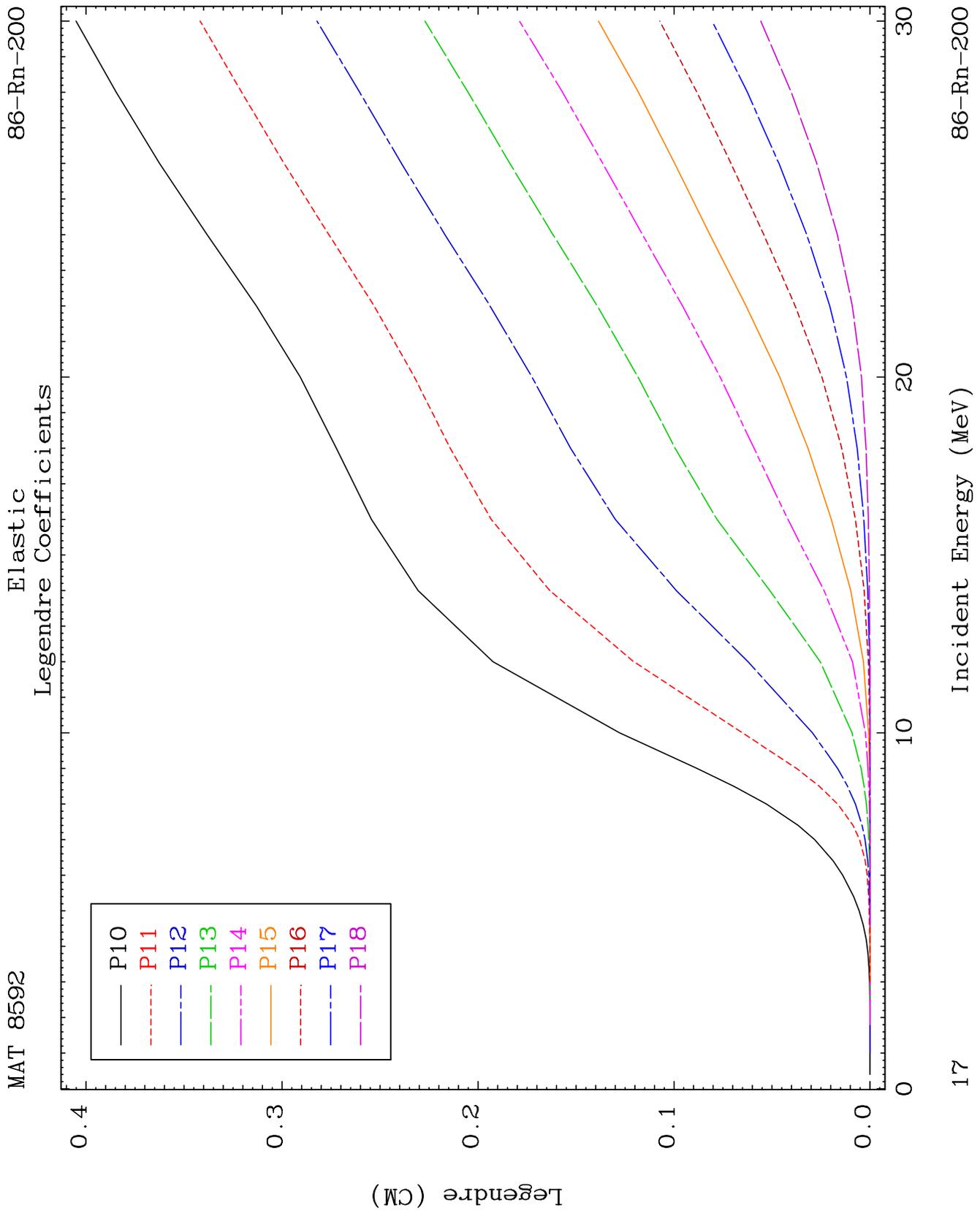
(n,He3) Levels
293 Kelvin Cross Sections

86-Rn-200





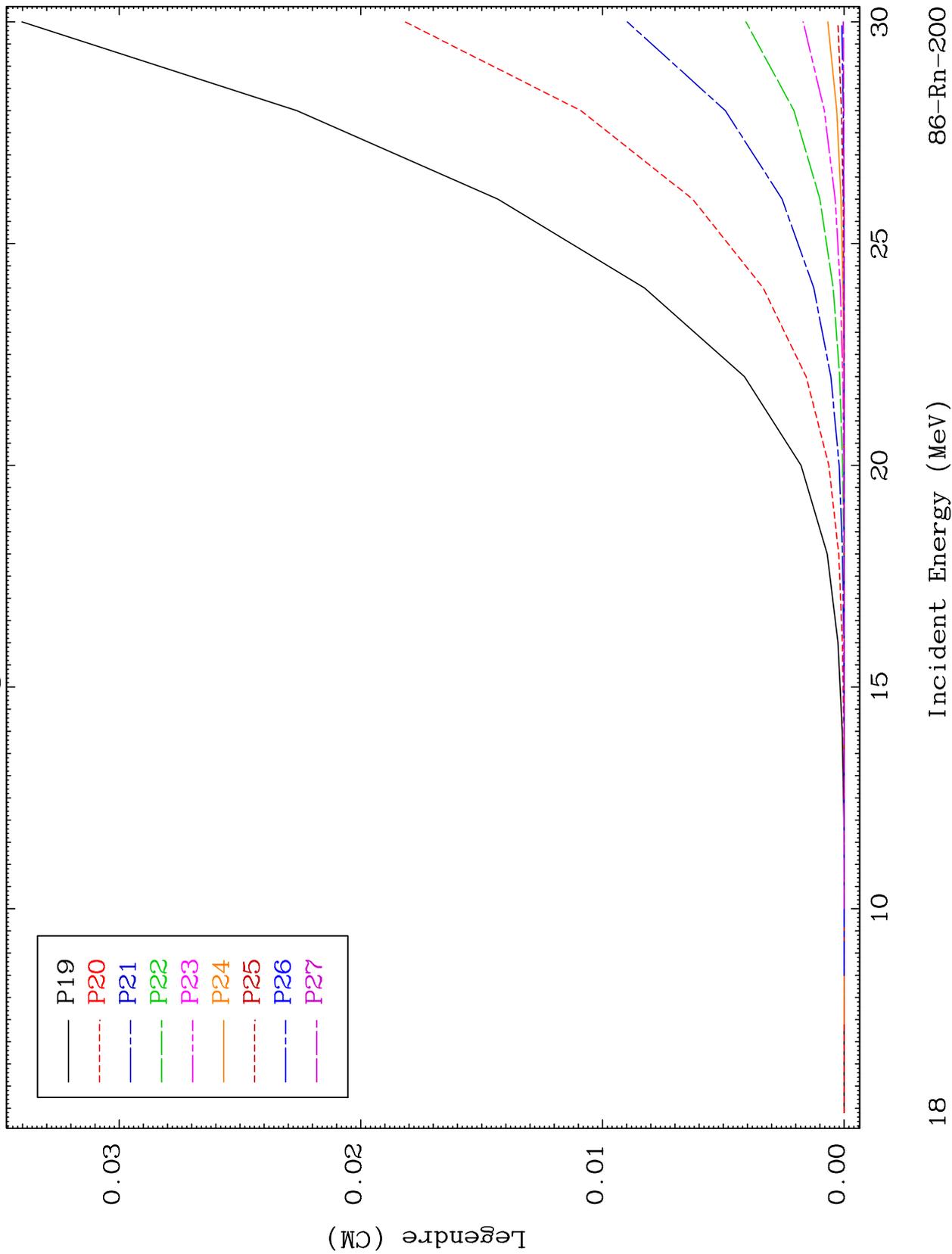




MAT 8592

Elastic Legendre Coefficients

86-Rn-200



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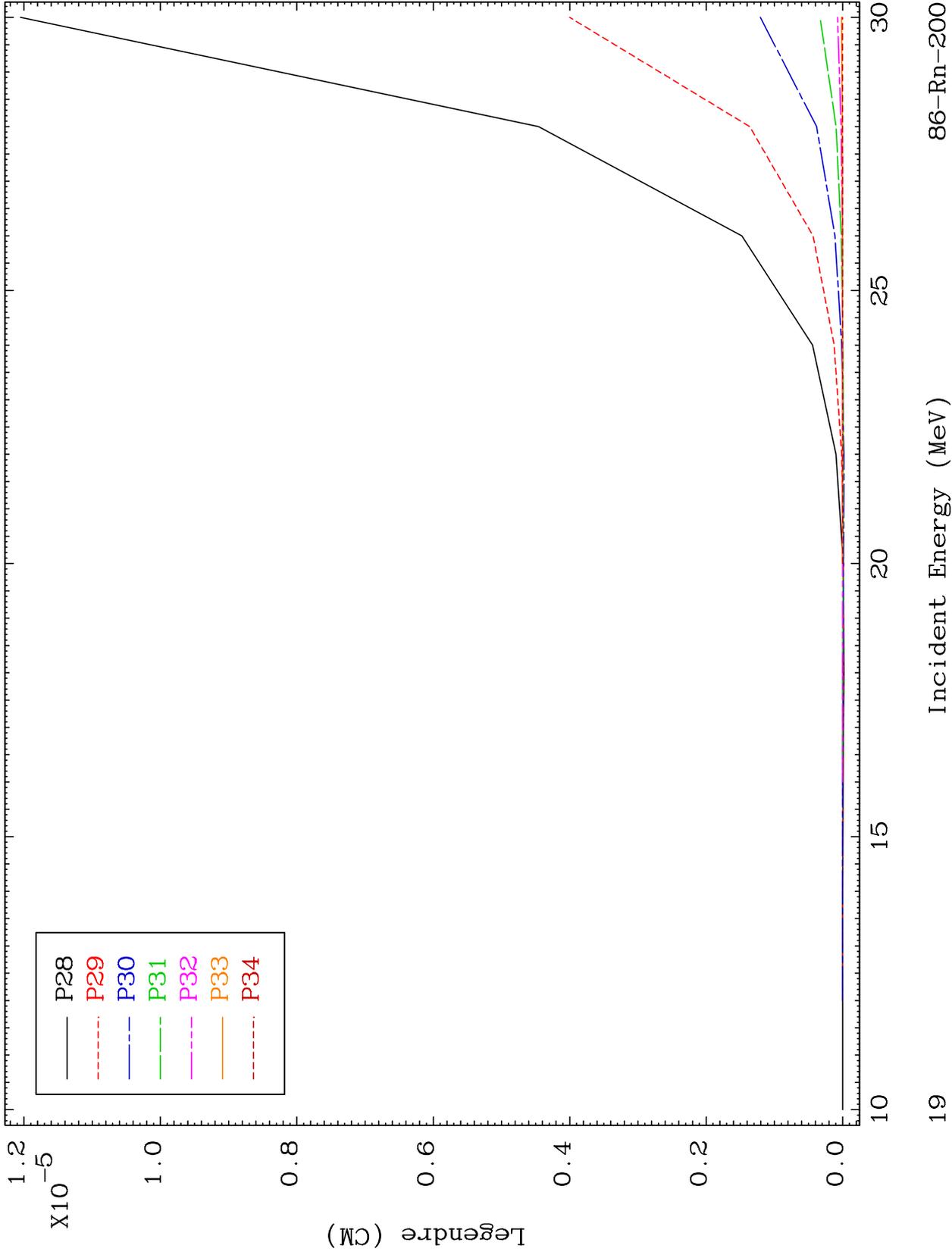
Incident Energy (MeV)

86-Rn-200

MAT 8592

Elastic
Legendre Coefficients

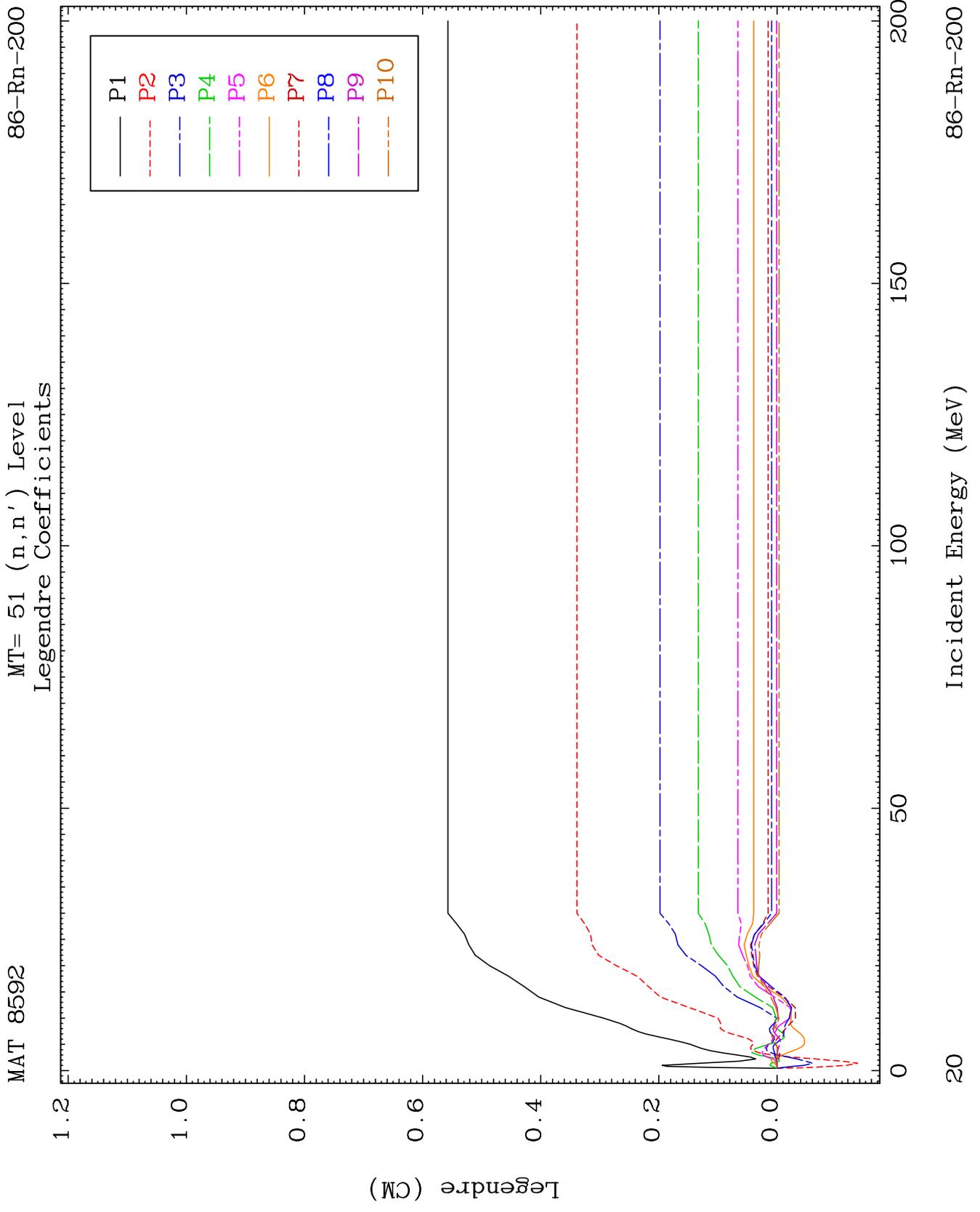
86-Rn-200



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Incident Energy (MeV)

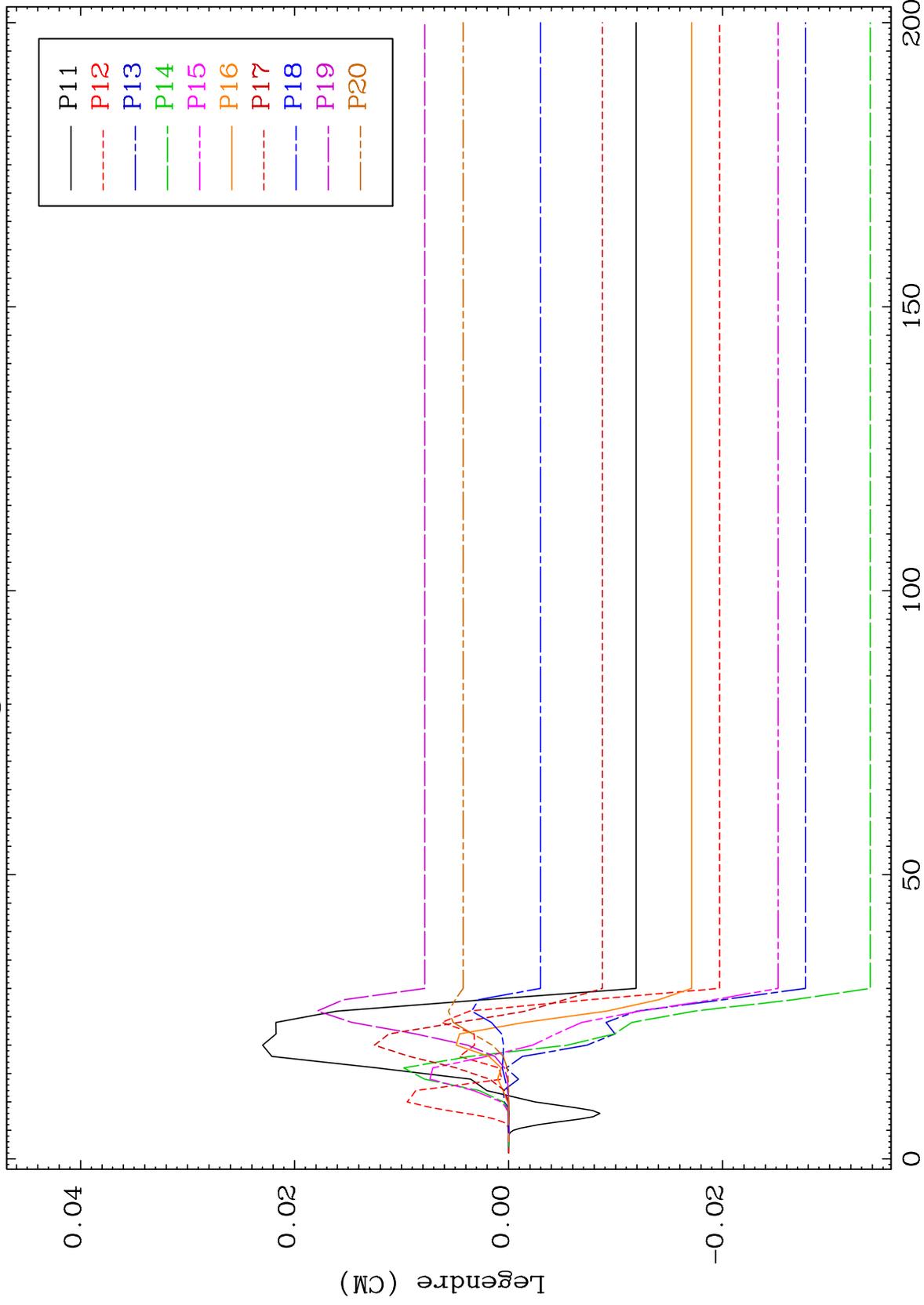
86-Rn-200



MAT 8592

MT= 51 (n,n') Level
Legendre Coefficients

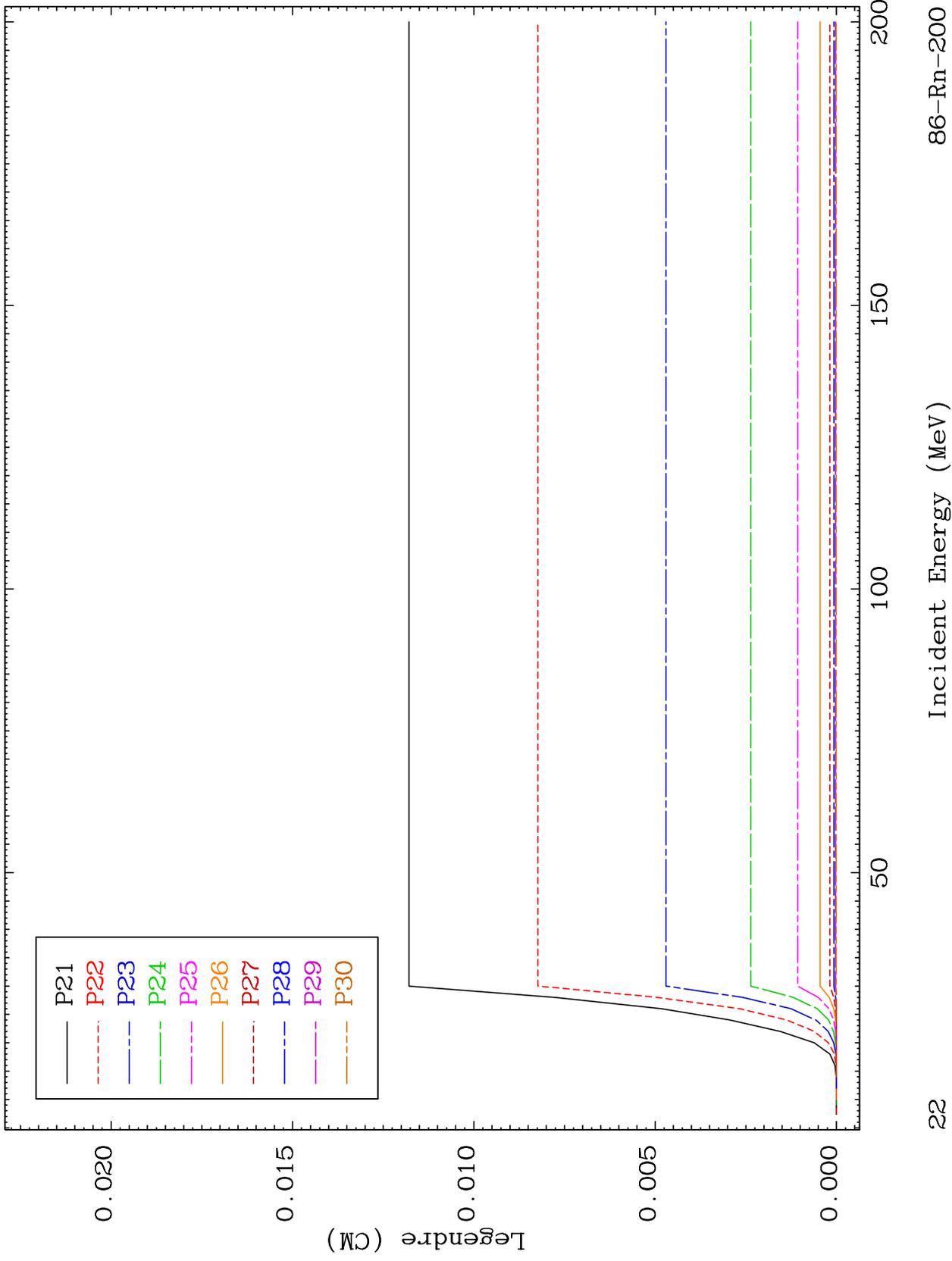
86-Rn-200



21

Incident Energy (MeV)

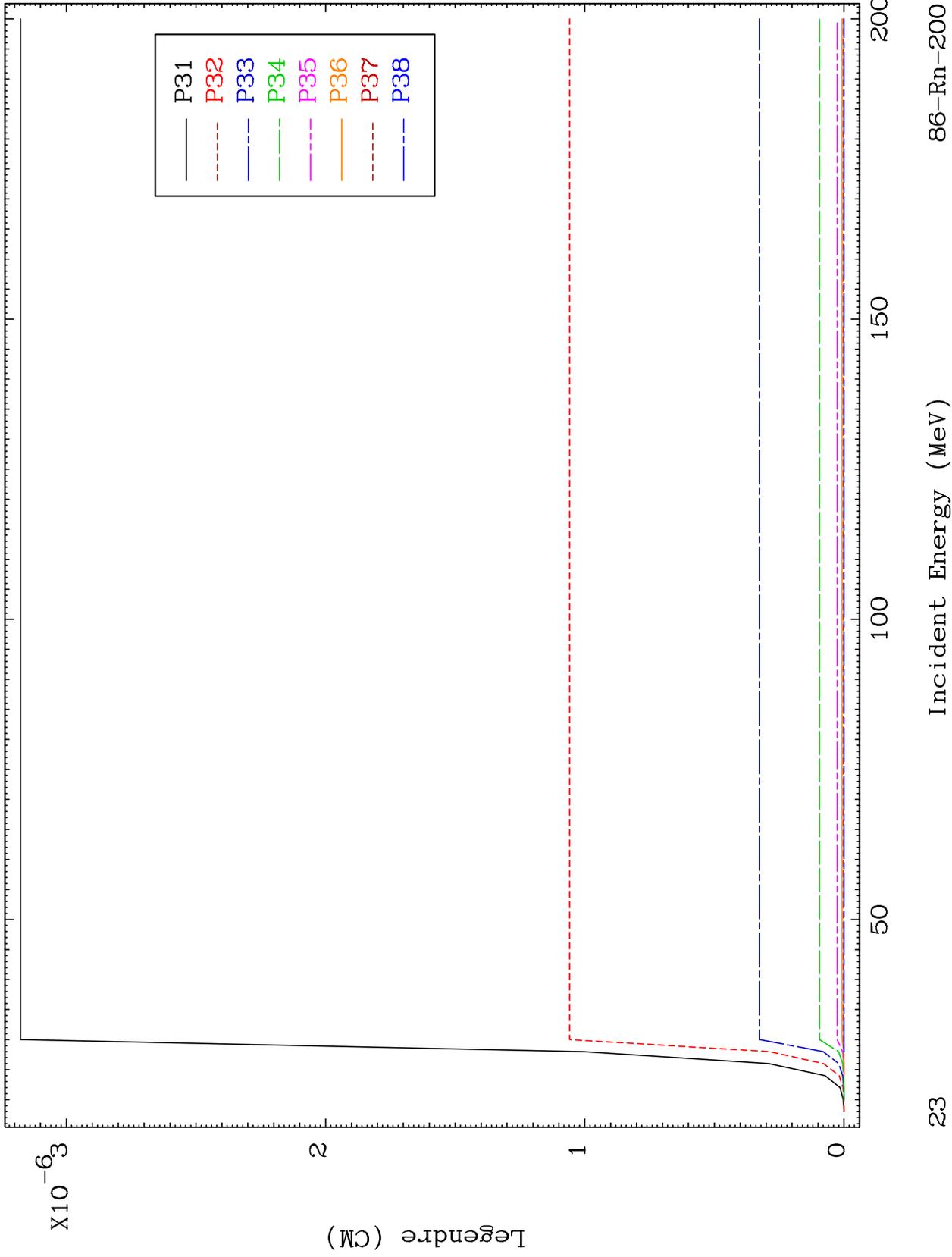
86-Rn-200



MAT 8592

MT= 51 (n,n') Level
Legendre Coefficients

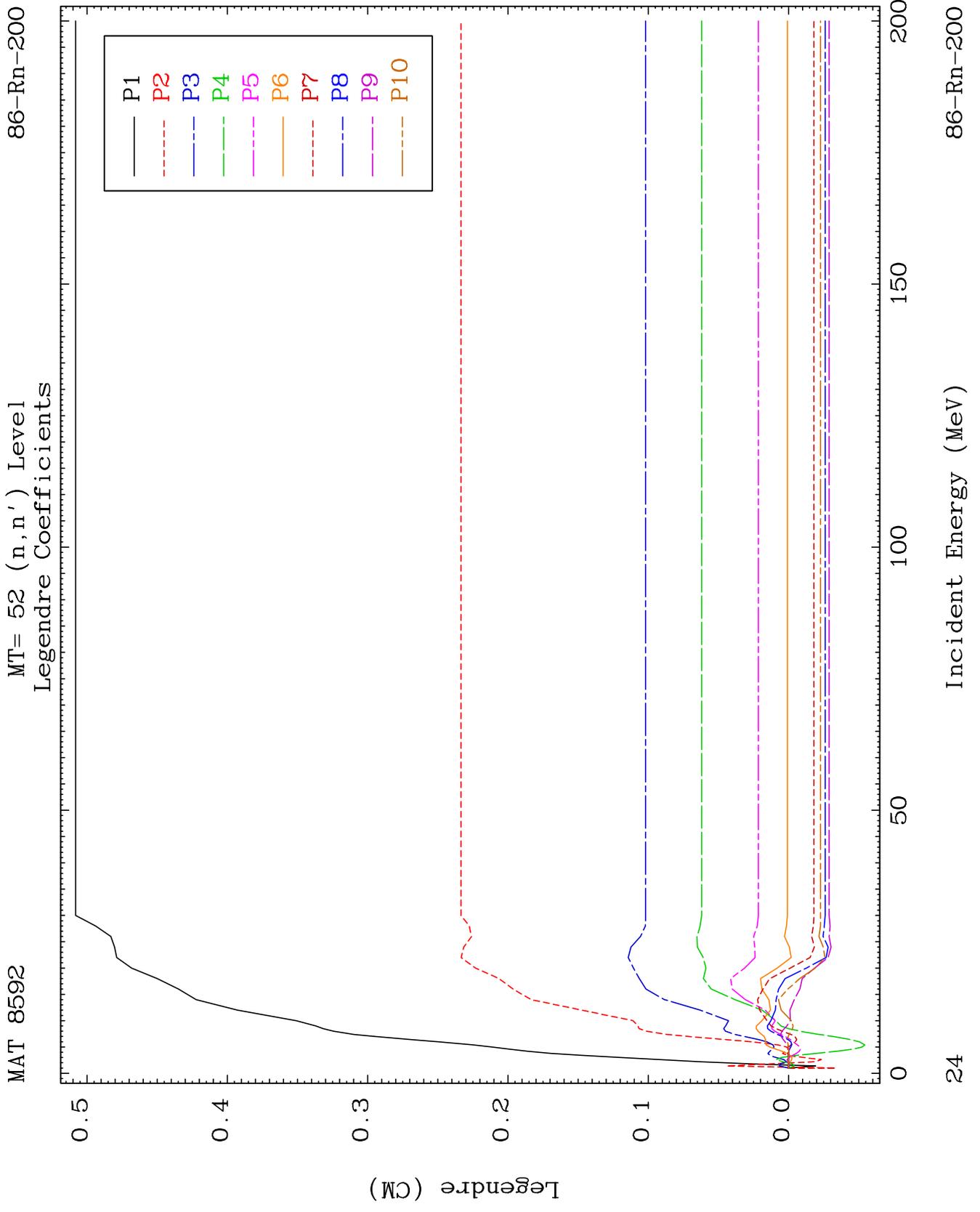
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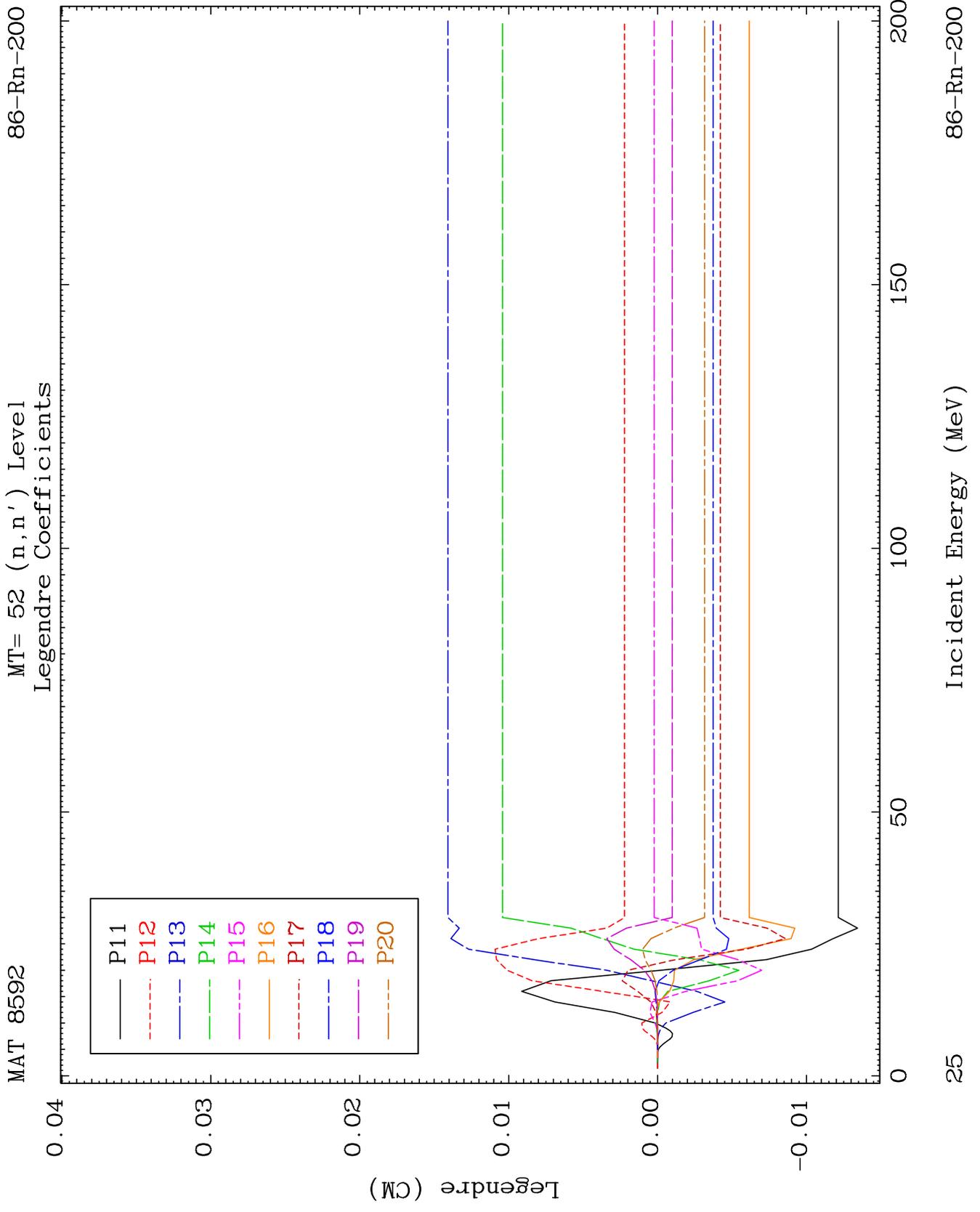


23

Incident Energy (MeV)

86-Rn-200

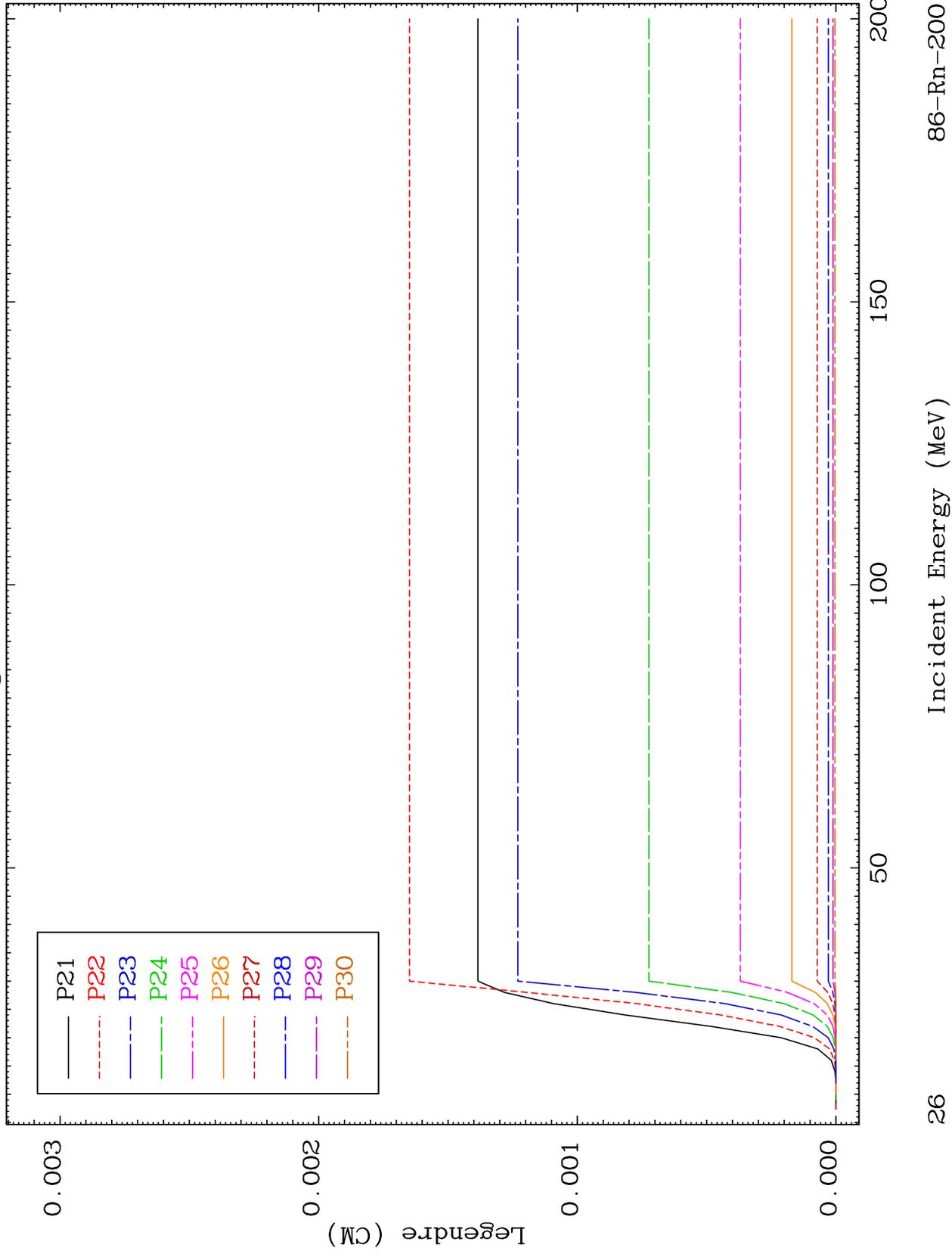




MAT 8592

MT= 52 (n,n') Level
Legendre Coefficients

86-Rn-200

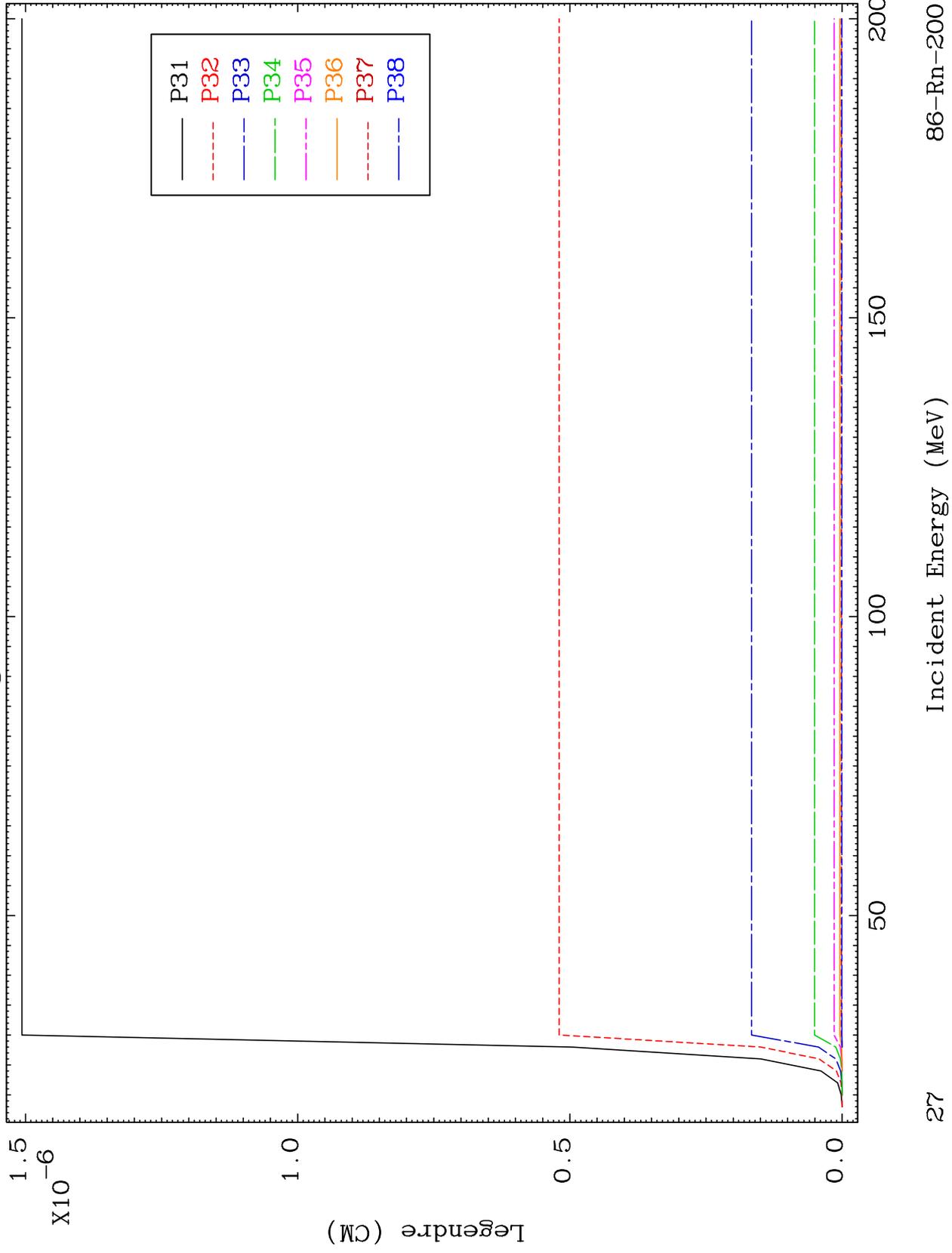


26

MAT 8592

MT= 52 (n,n') Level
Legendre Coefficients

86-Rn-200

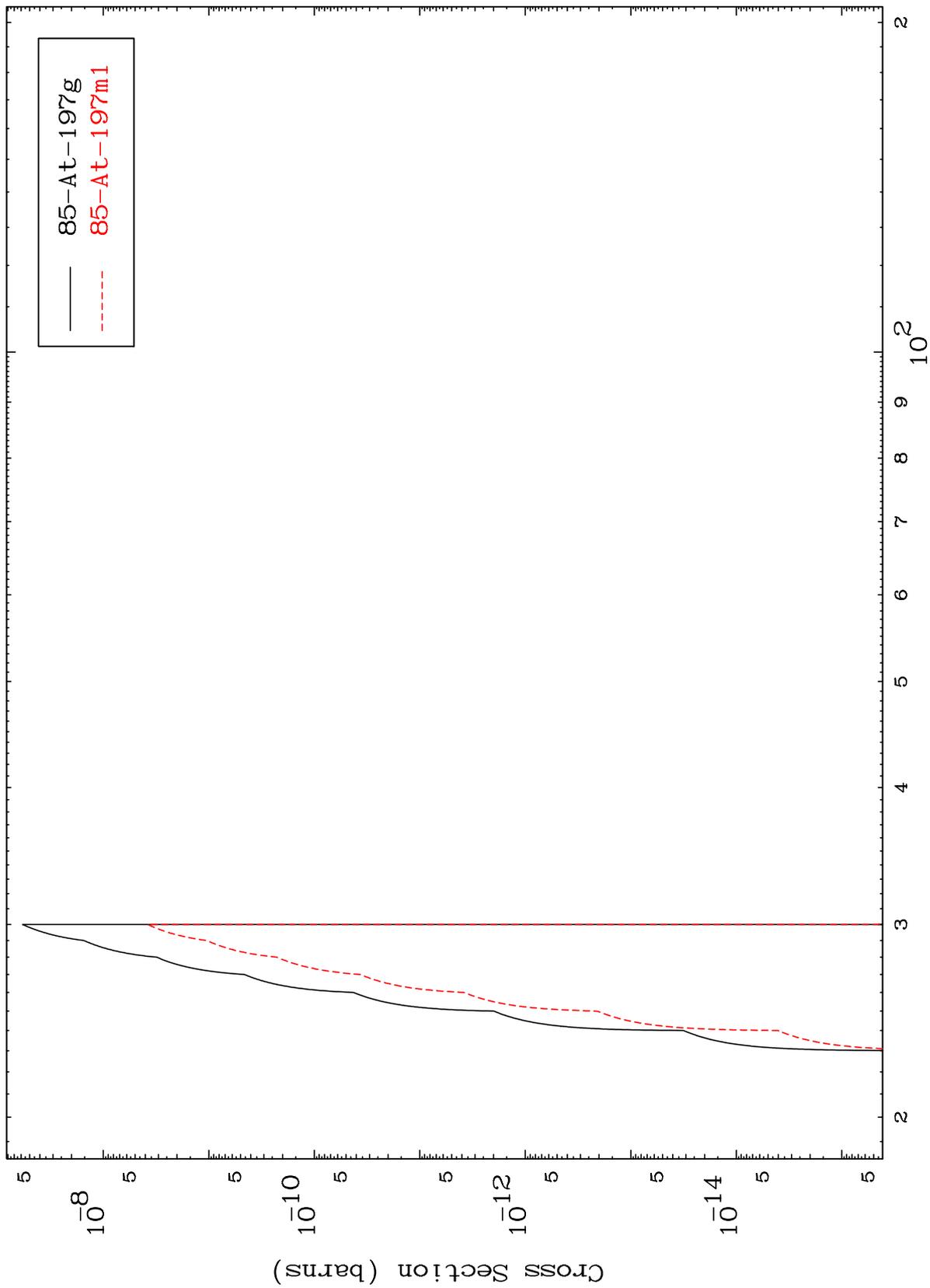


MAT 8592

(n,2n) d

86-Rn-200

Radionuclide Production Cross Section

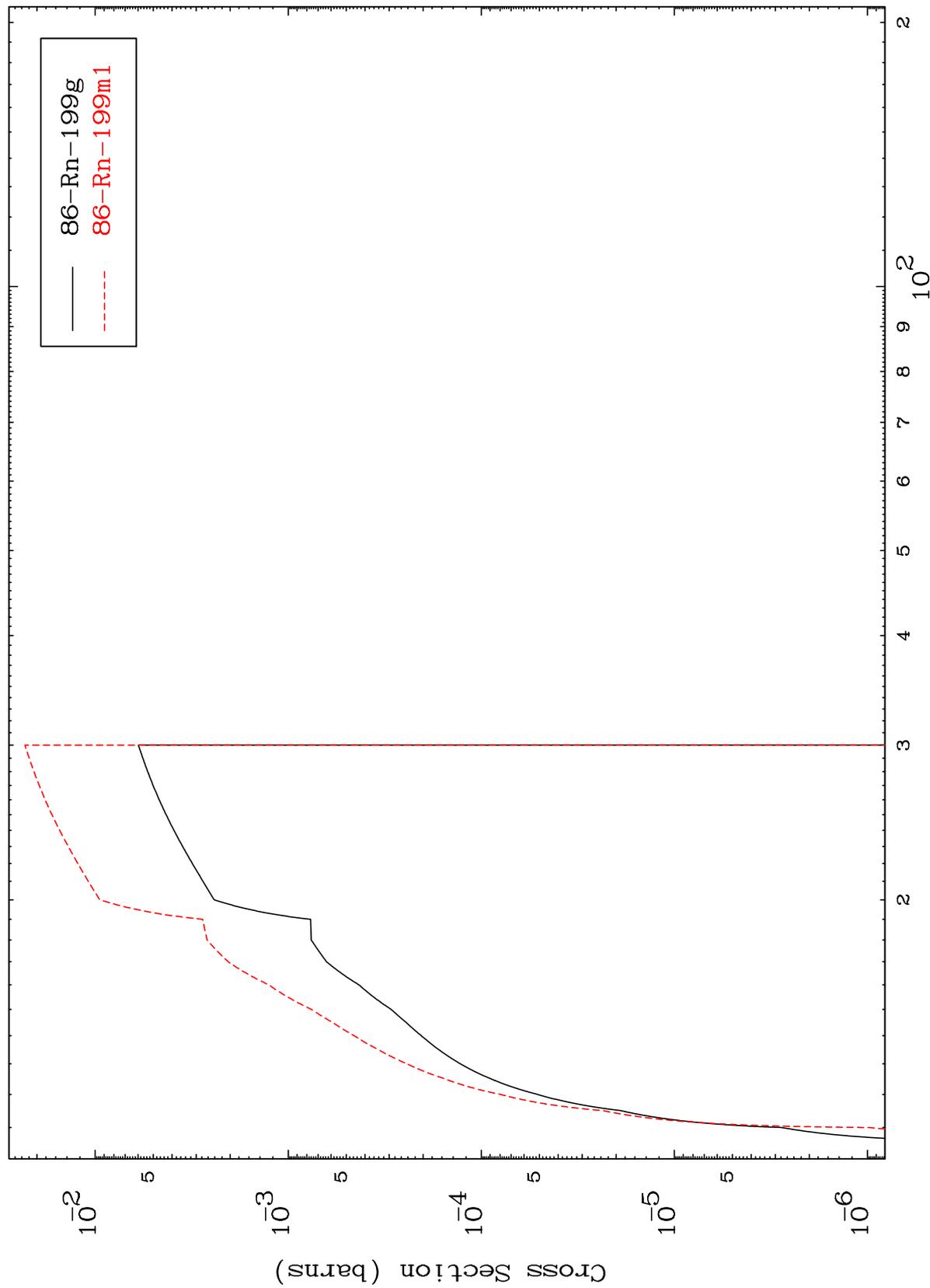


28

Incident Energy (MeV)

86-Rn-200

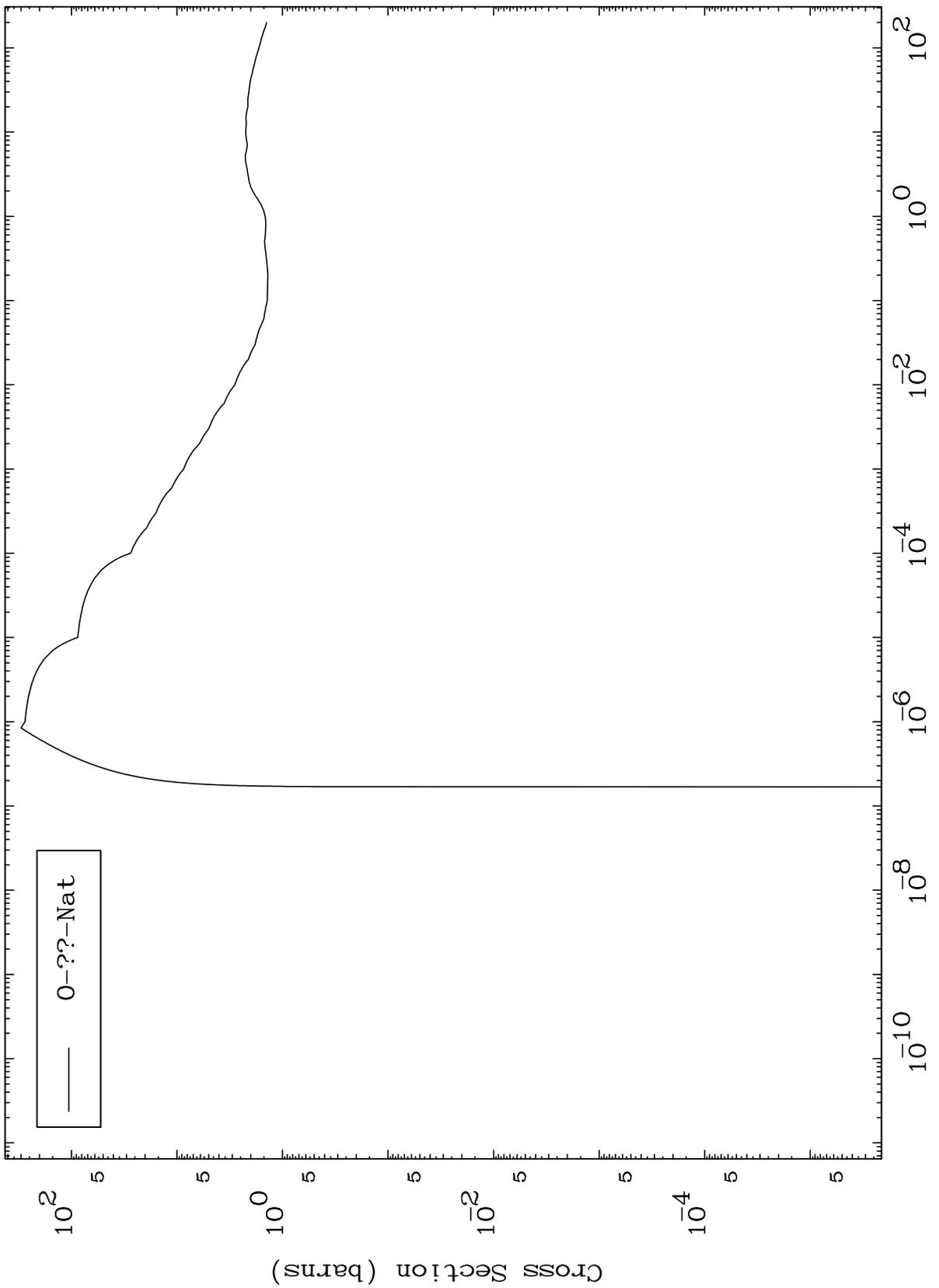
(n,2n)
Radionuclide Production Cross Section



MAT 8592

86-Rn-200

Fission
Radionuclide Production Cross Section

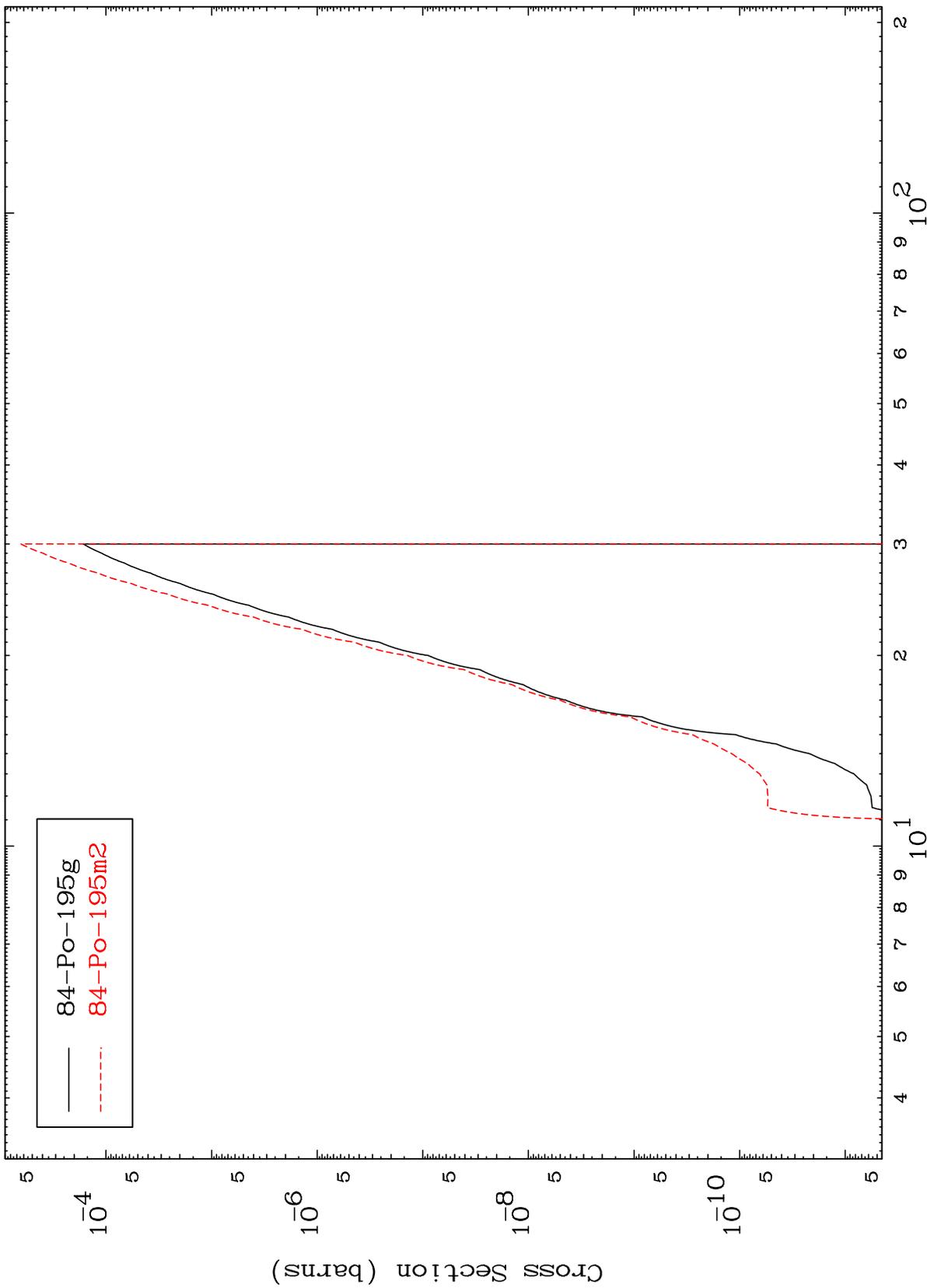


MAT 8592

$(n,2n) \alpha$

86-Rn-200

Radionuclide Production Cross Section



84-Po-195g
84-Po-195m2

Incident Energy (MeV)

86-Rn-200

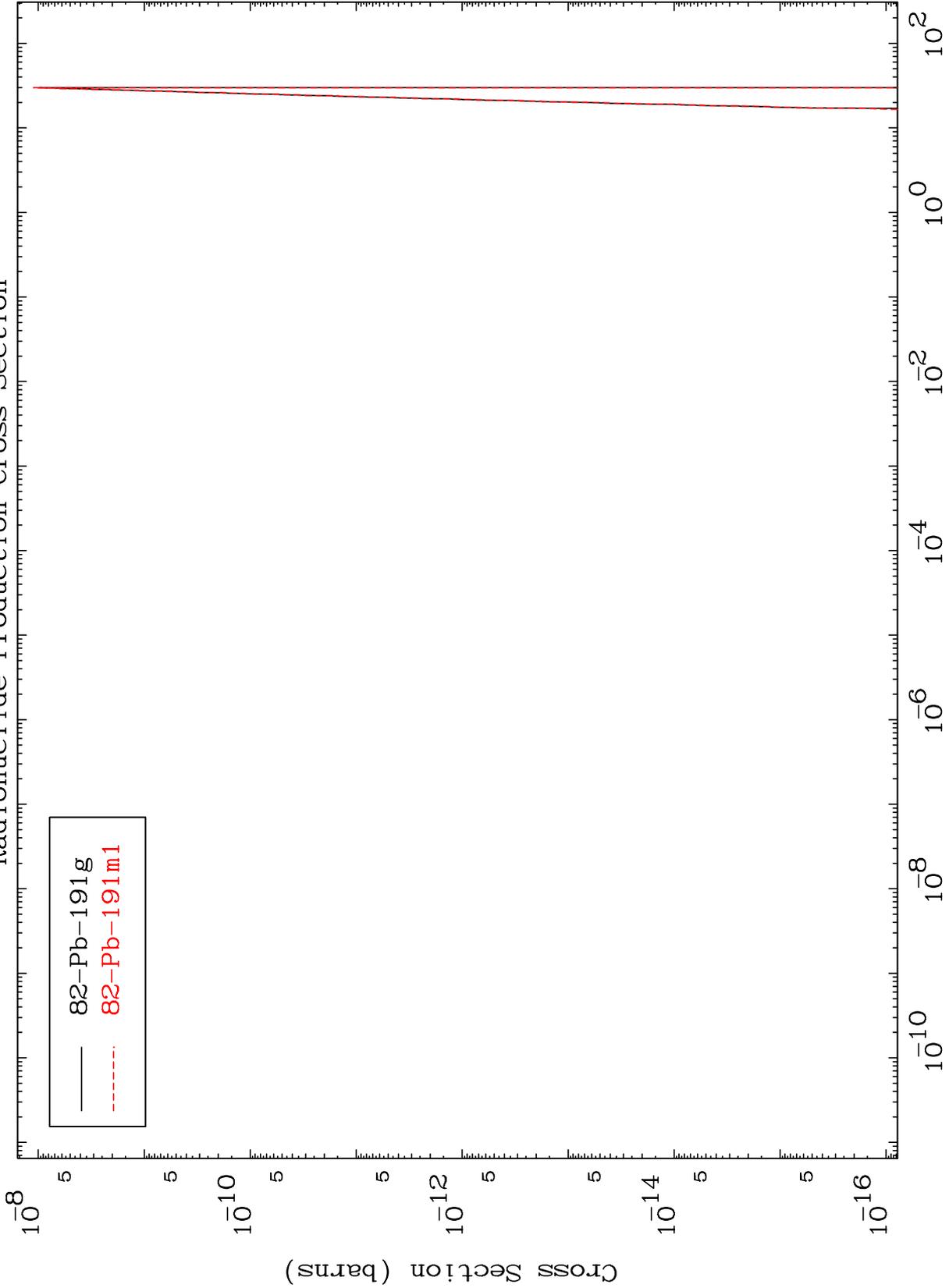
31

MAT 8592

$(n,2n) 2\alpha$

86-Rn-200

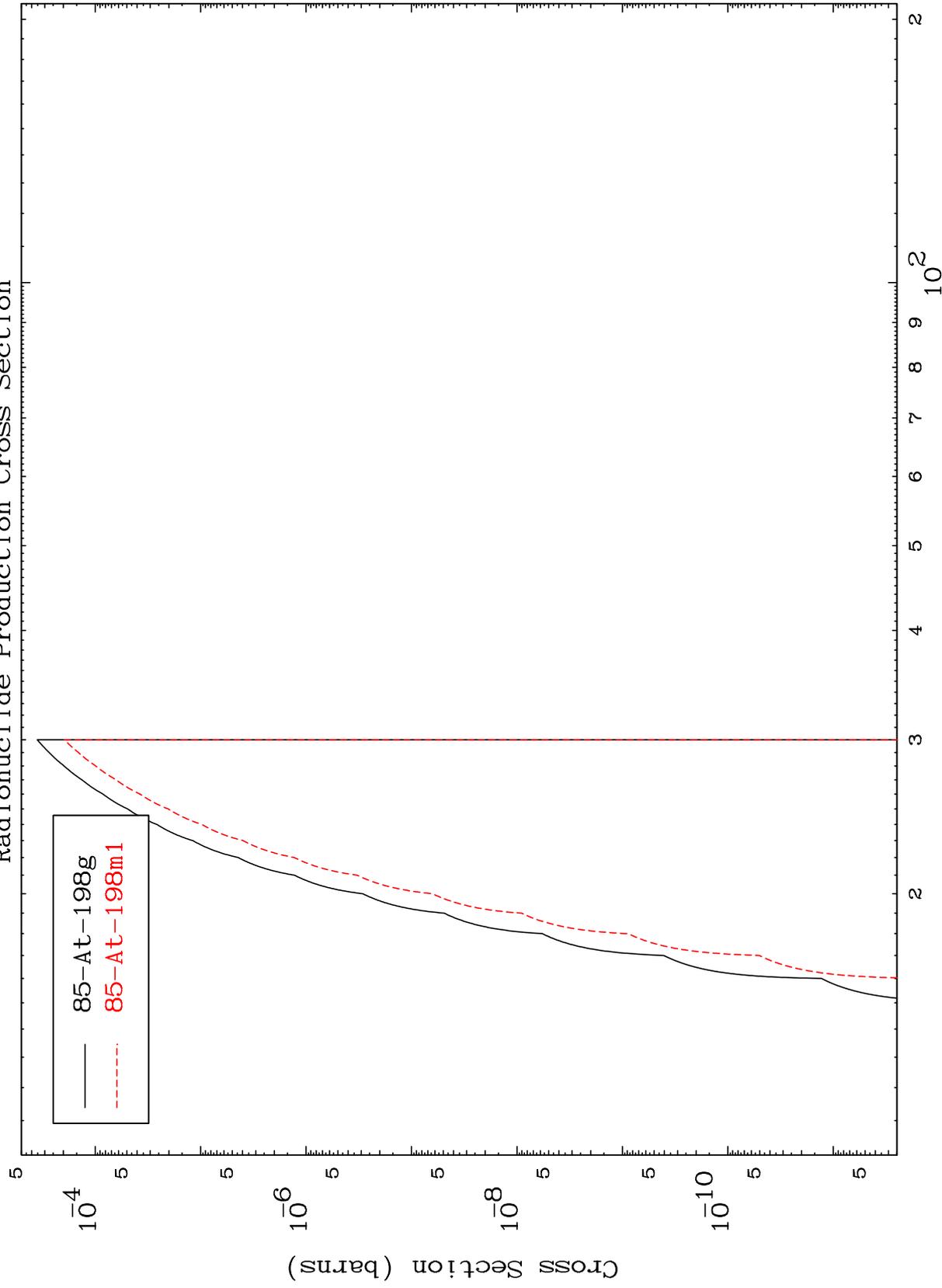
Radionuclide Production Cross Section



MAT 8592

86-Rn-200

(n,n') d
Radionuclide Production Cross Section



86-Rn-200

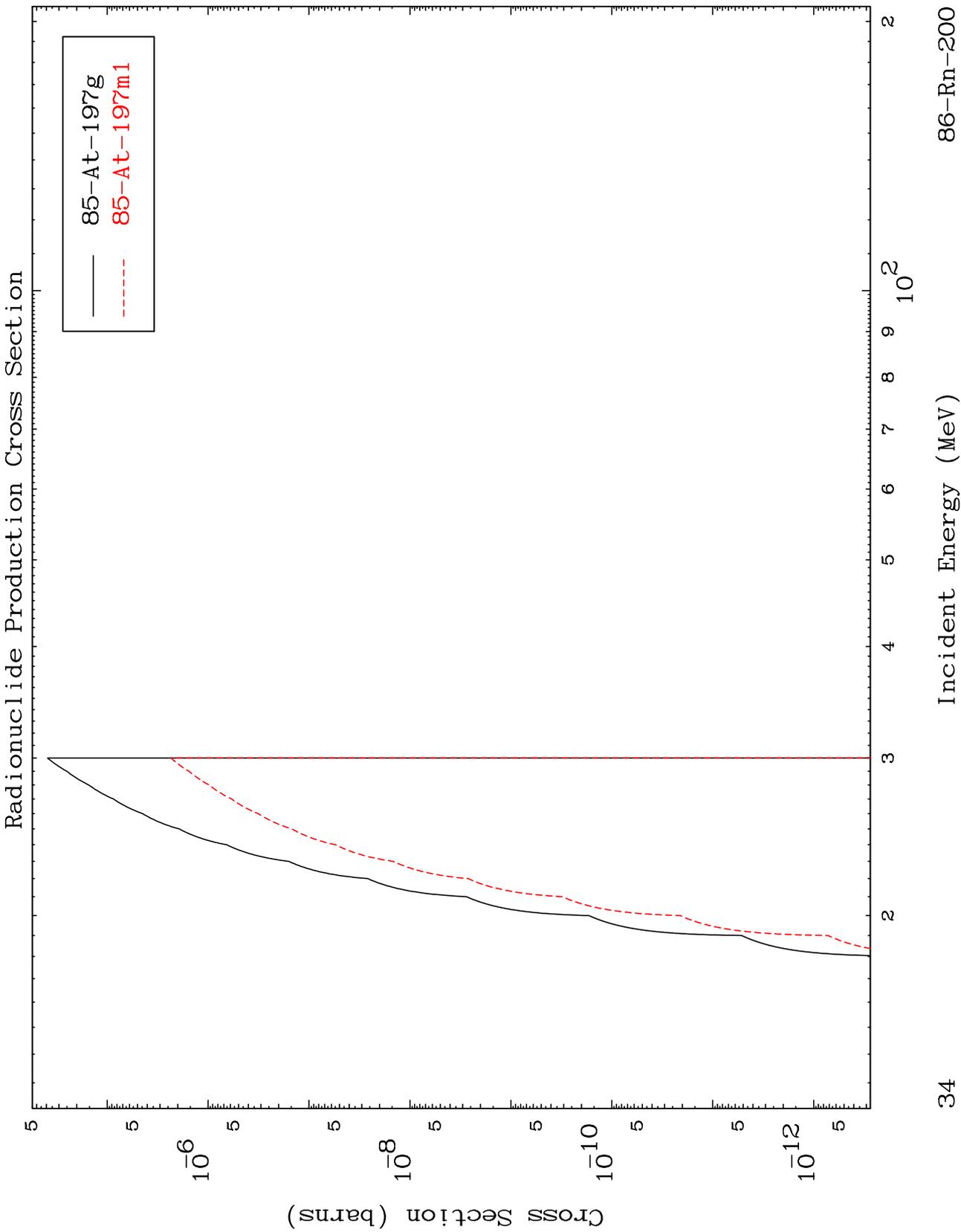
Incident Energy (MeV)

33

MAT 8592

(n,n') t

86-Rn-200



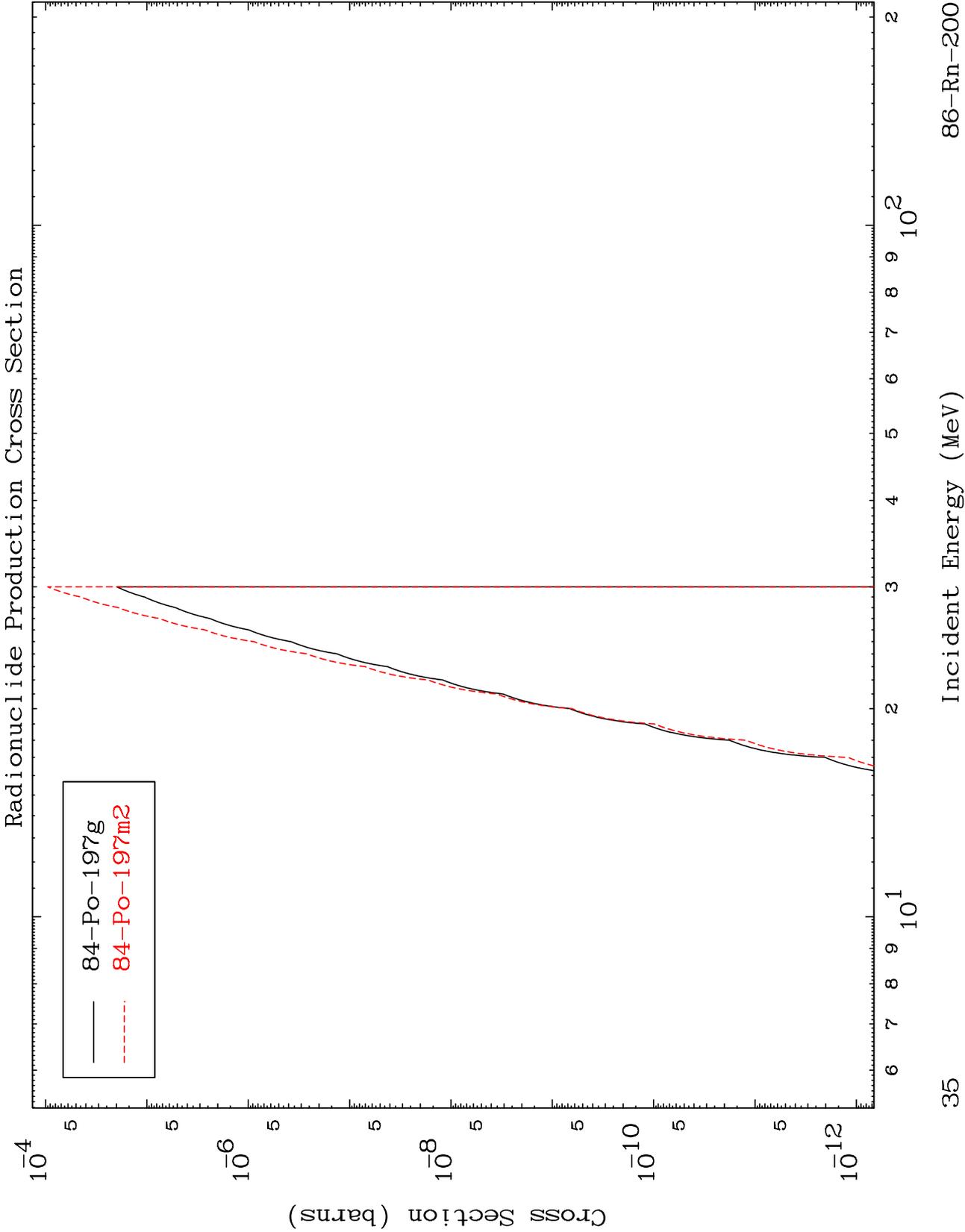
34

86-Rn-200

MAT 8592

(n,n') He-3

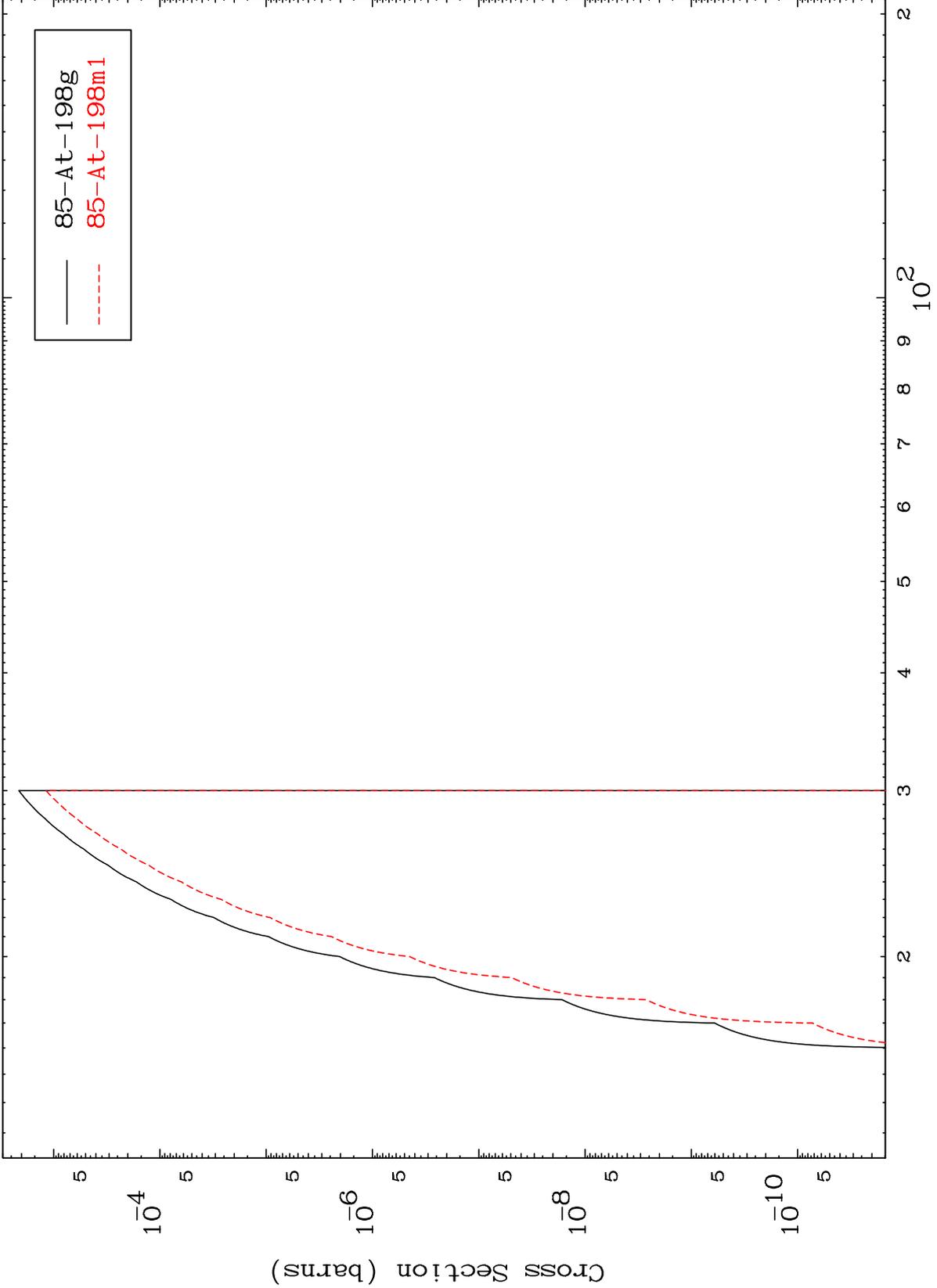
86-Rn-200



35

86-Rn-200

Radionuclide Production Cross Section

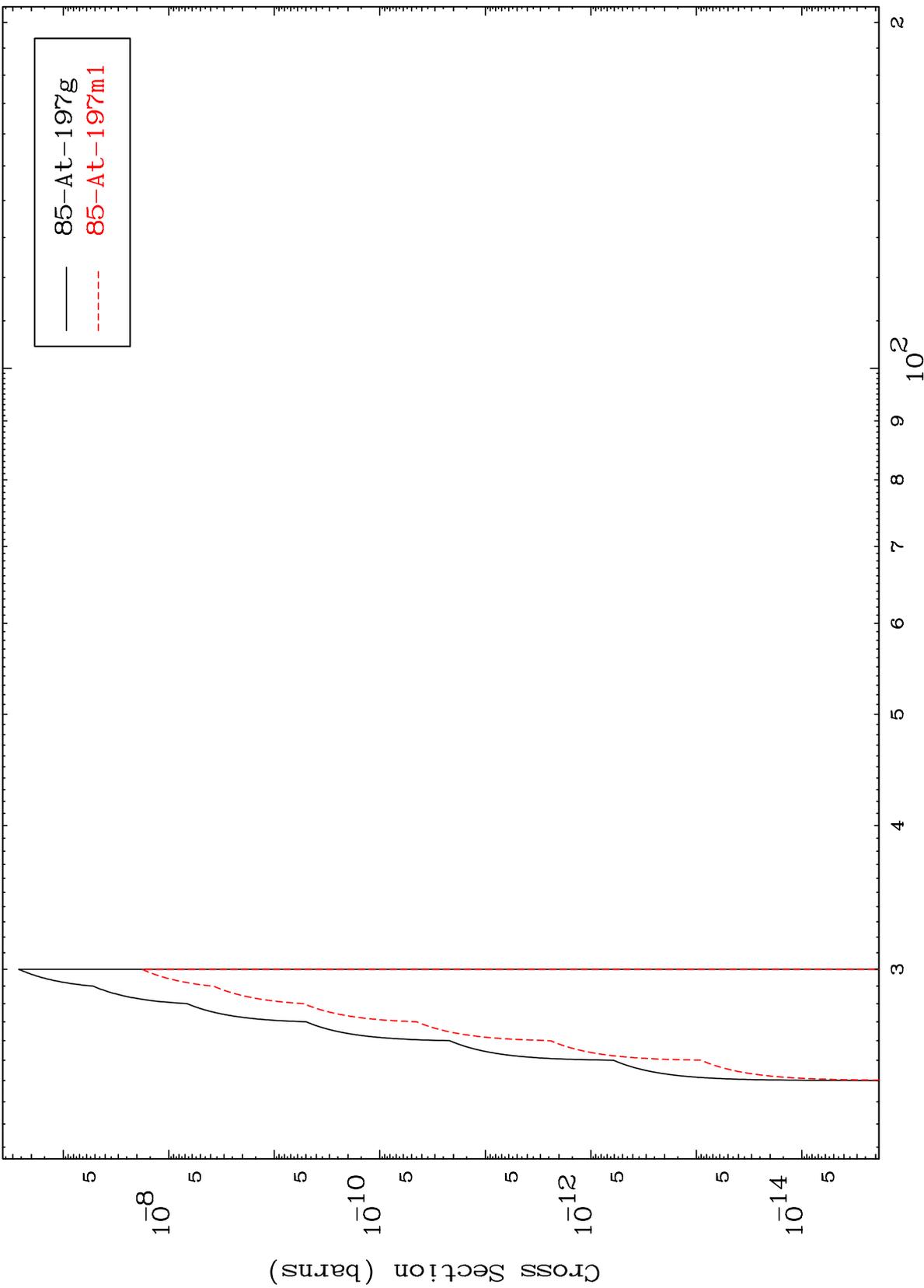


MAT 8592

(n,3n) p

86-Rn-200

Radionuclide Production Cross Section



37

Incident Energy (MeV)

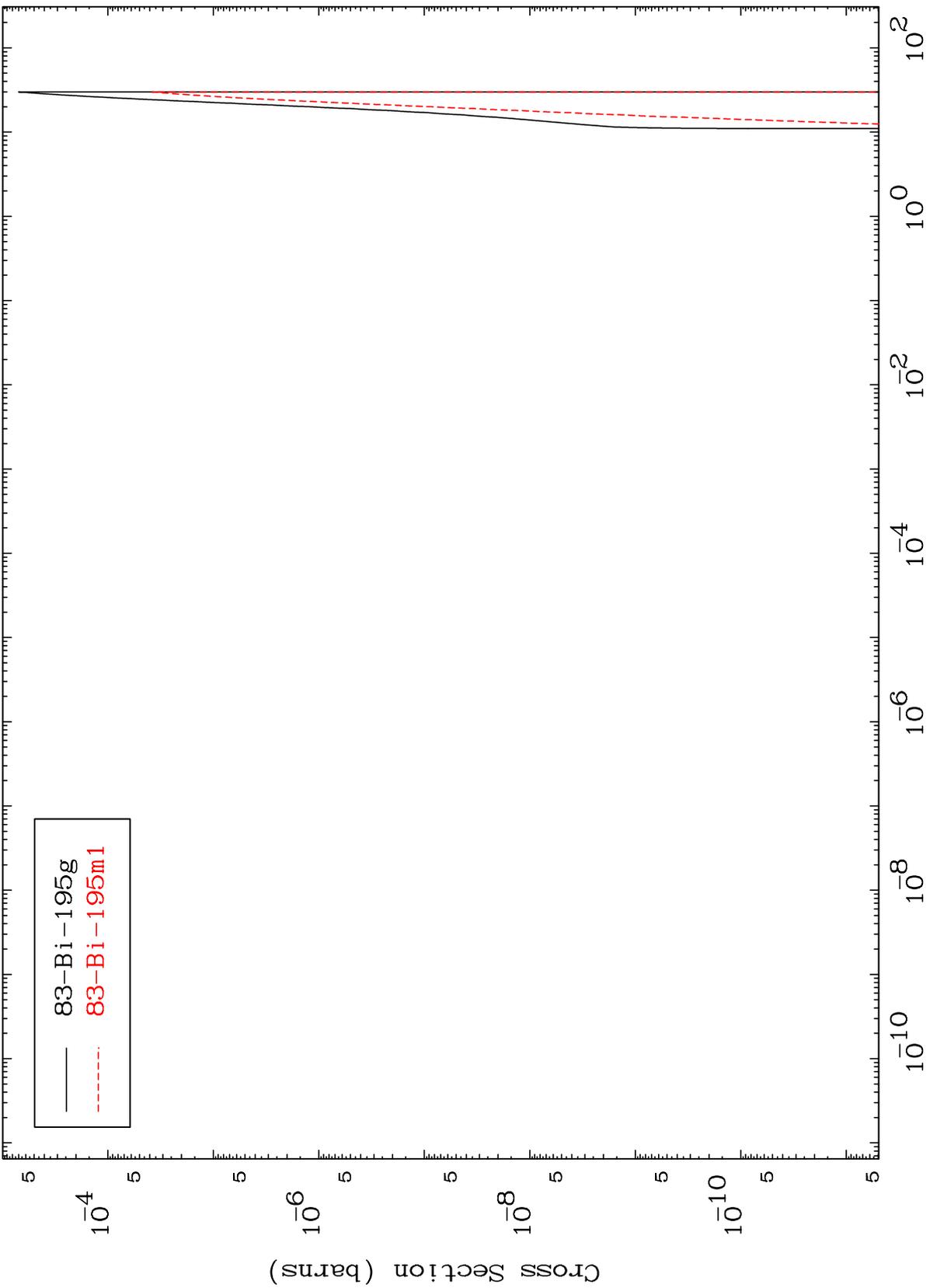
86-Rn-200

MAT 8592

(n,n') p α

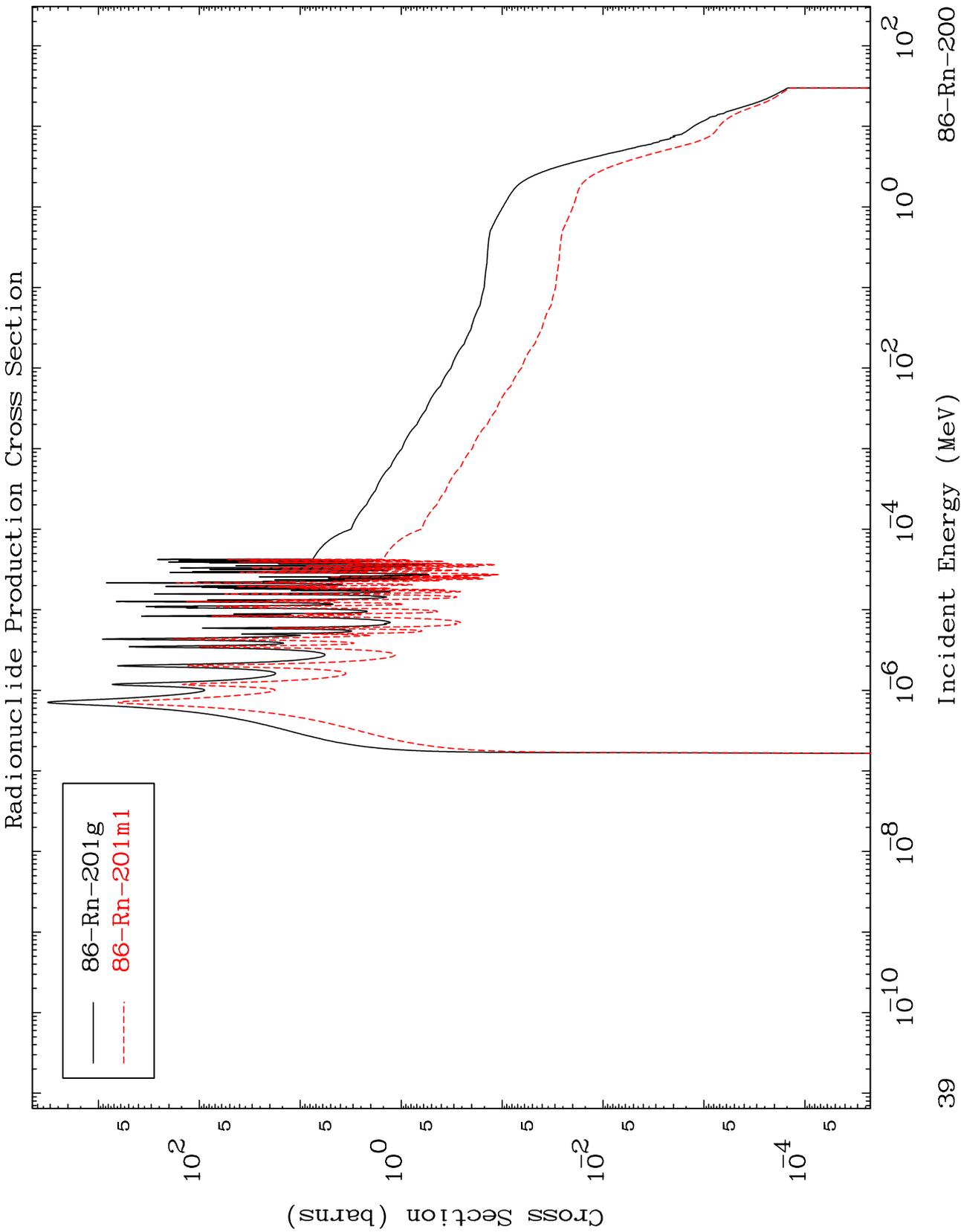
86-Rn-200

Radionuclide Production Cross Section



MAT 8592

86-Rn-200

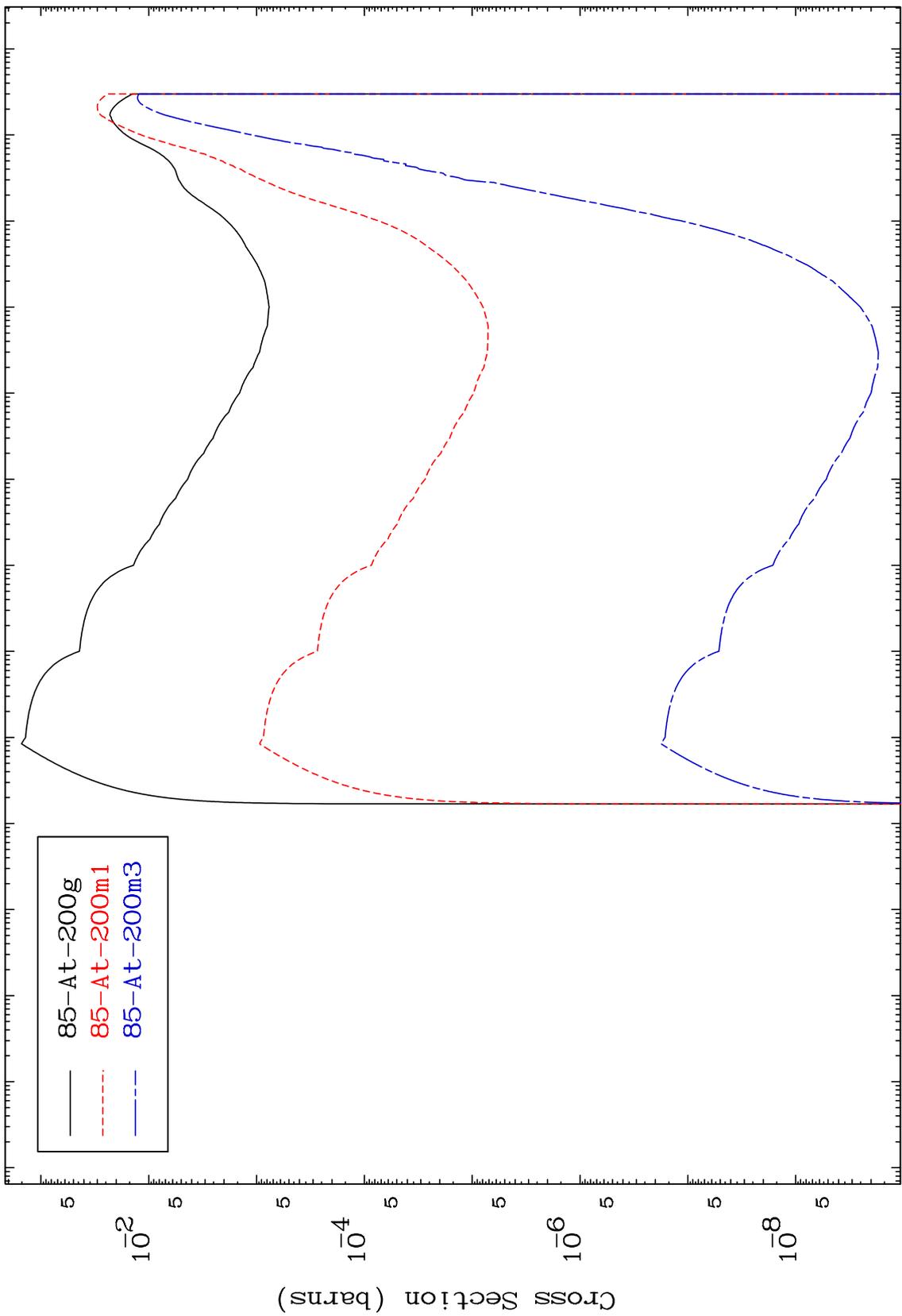


— 86-Rn-201 g
- - - 86-Rn-201 m1

MAT 8592

86-Rn-200

(n,p)
Radionuclide Production Cross Section



86-Rn-200

10⁻²

10⁻⁴

10⁻⁶

10⁻⁸

Incident Energy (MeV)

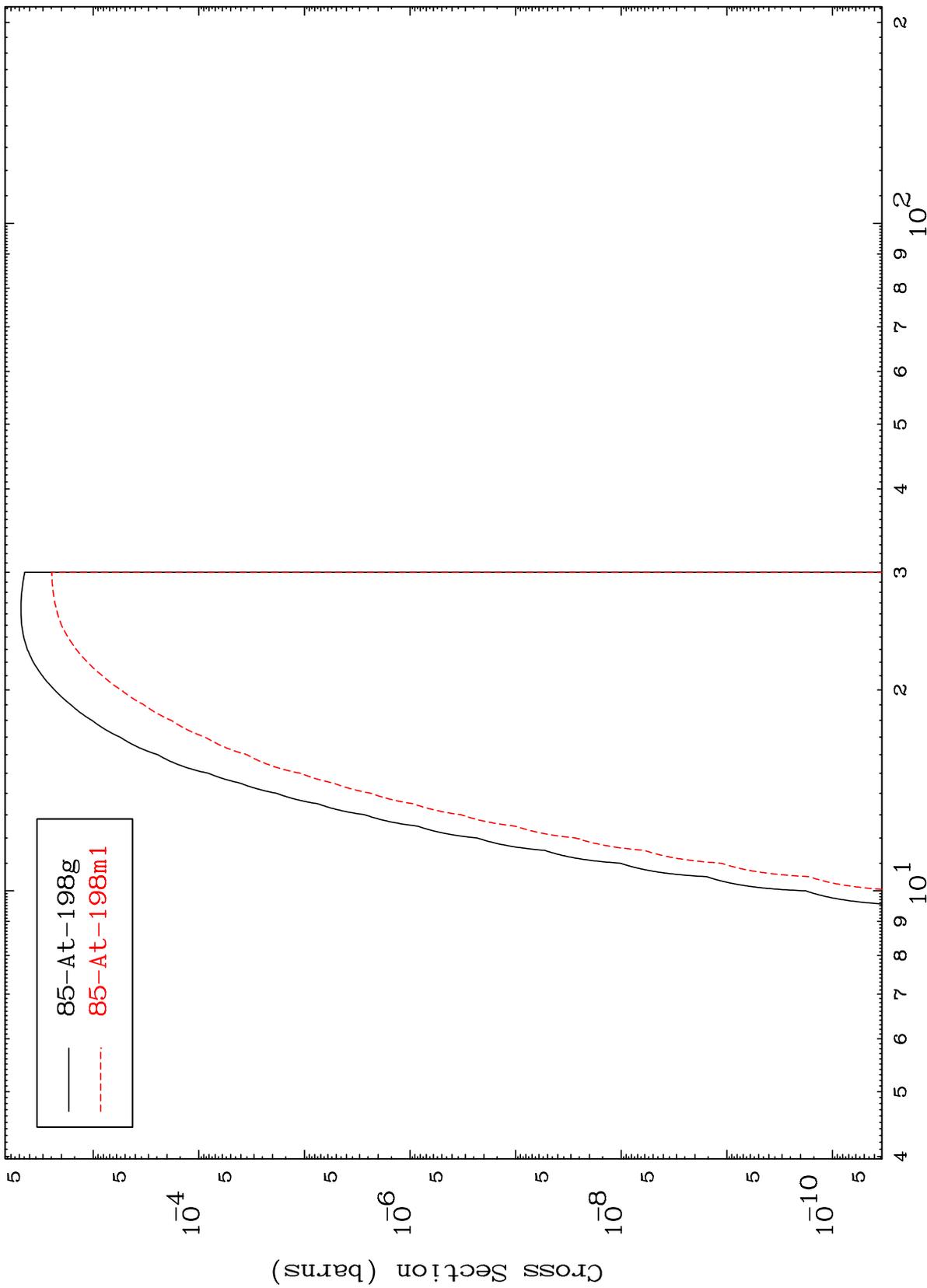
86-Rn-200

40

MAT 8592

86-Rn-200

(n, t)
Radionuclide Production Cross Section



86-Rn-200

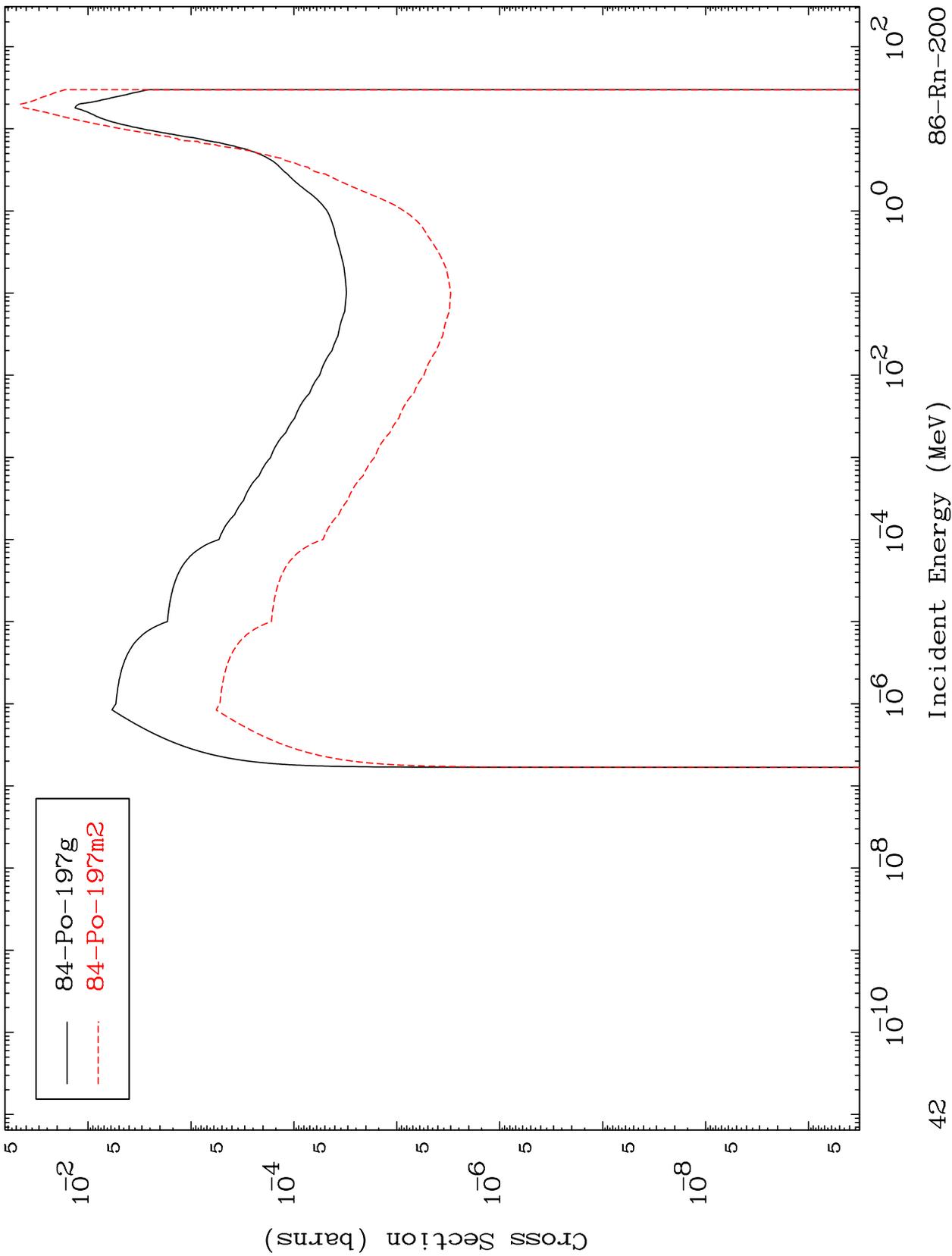
Incident Energy (MeV)

41

MAT 8592

86-Rn-200

Radionuclide Production Cross Section
(n, α)

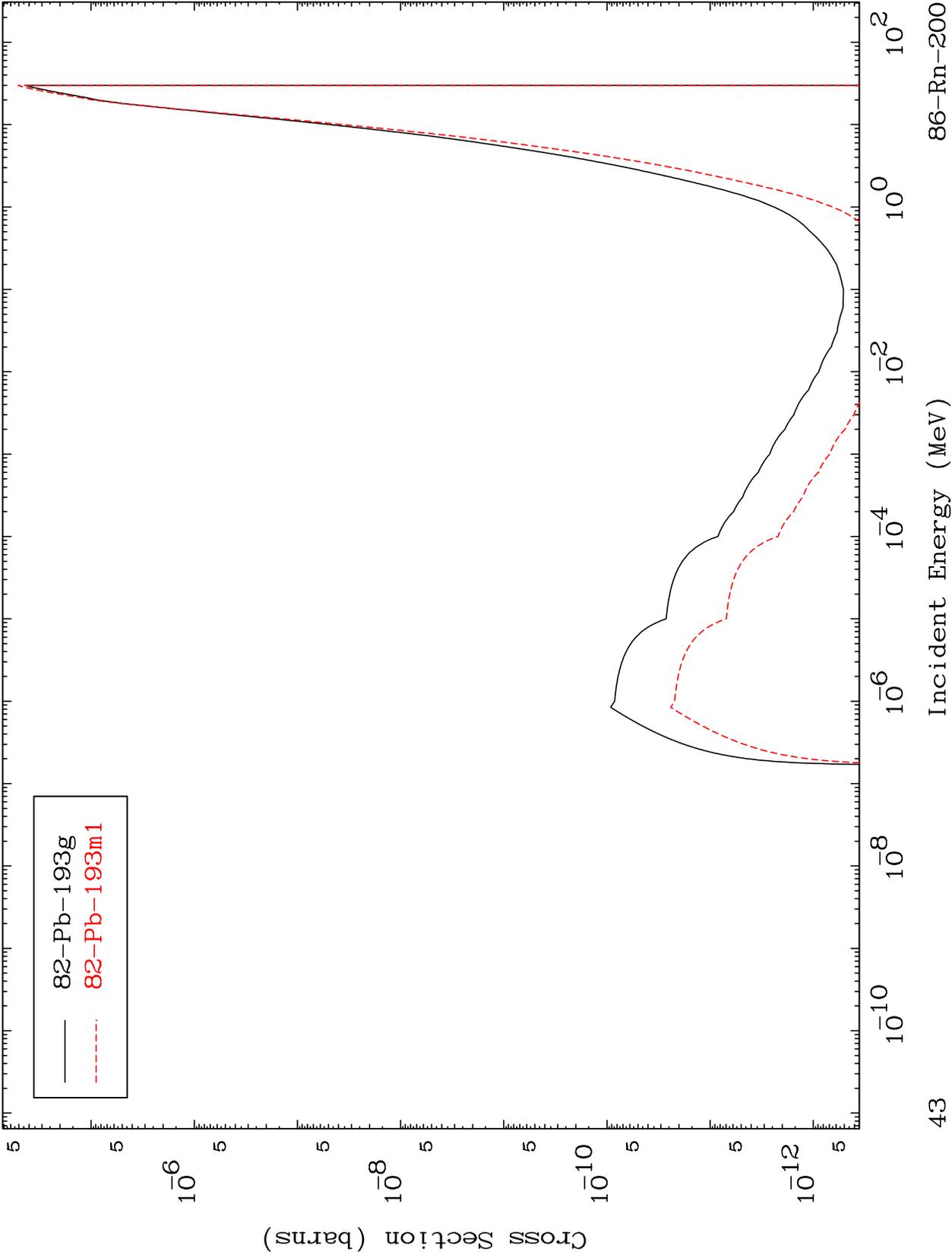


MAT 8592

(n,2 α)

86-Rn-200

Radionuclide Production Cross Section

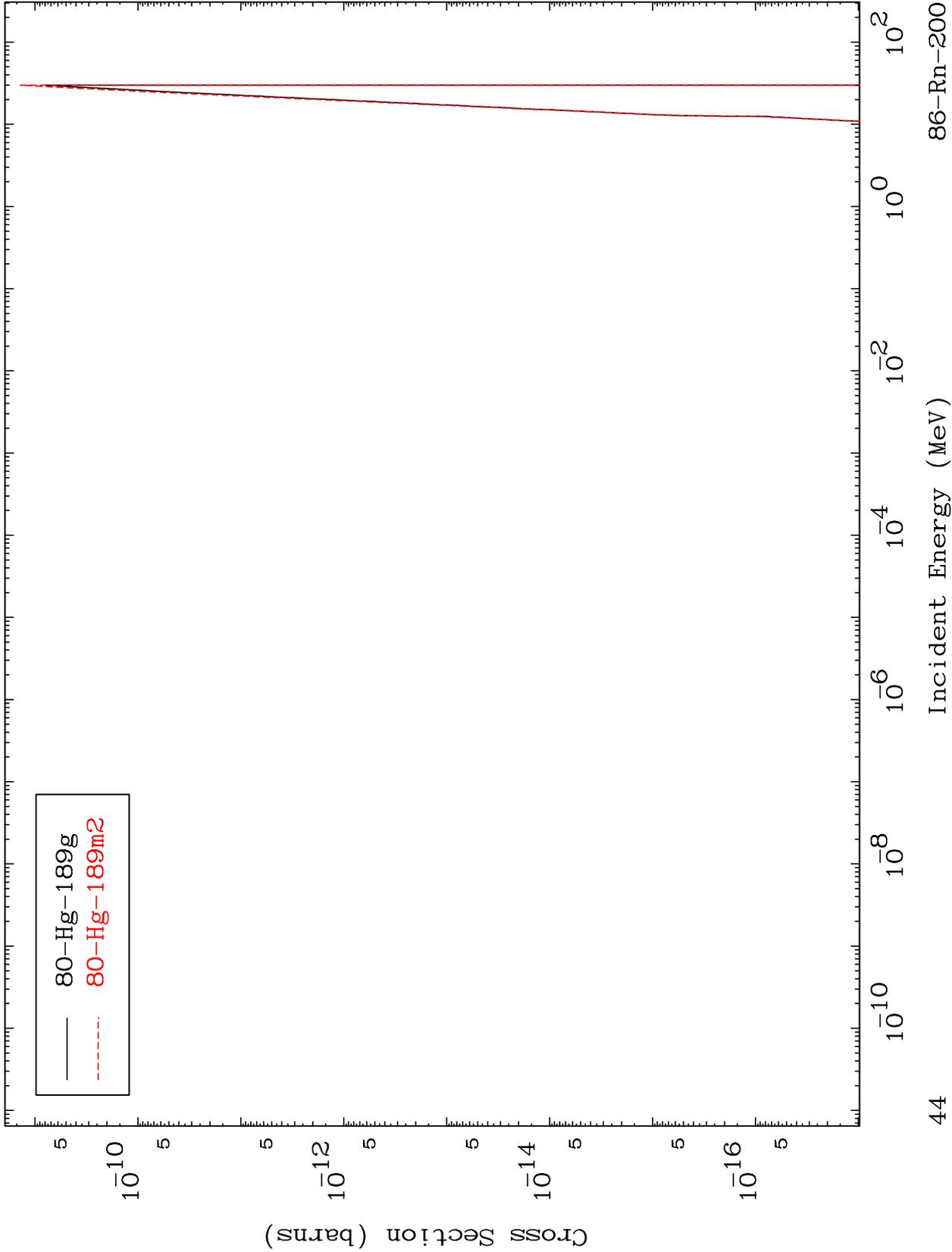


MAT 8592

(n, 3α)

86-Rn-200

Radionuclide Production Cross Section

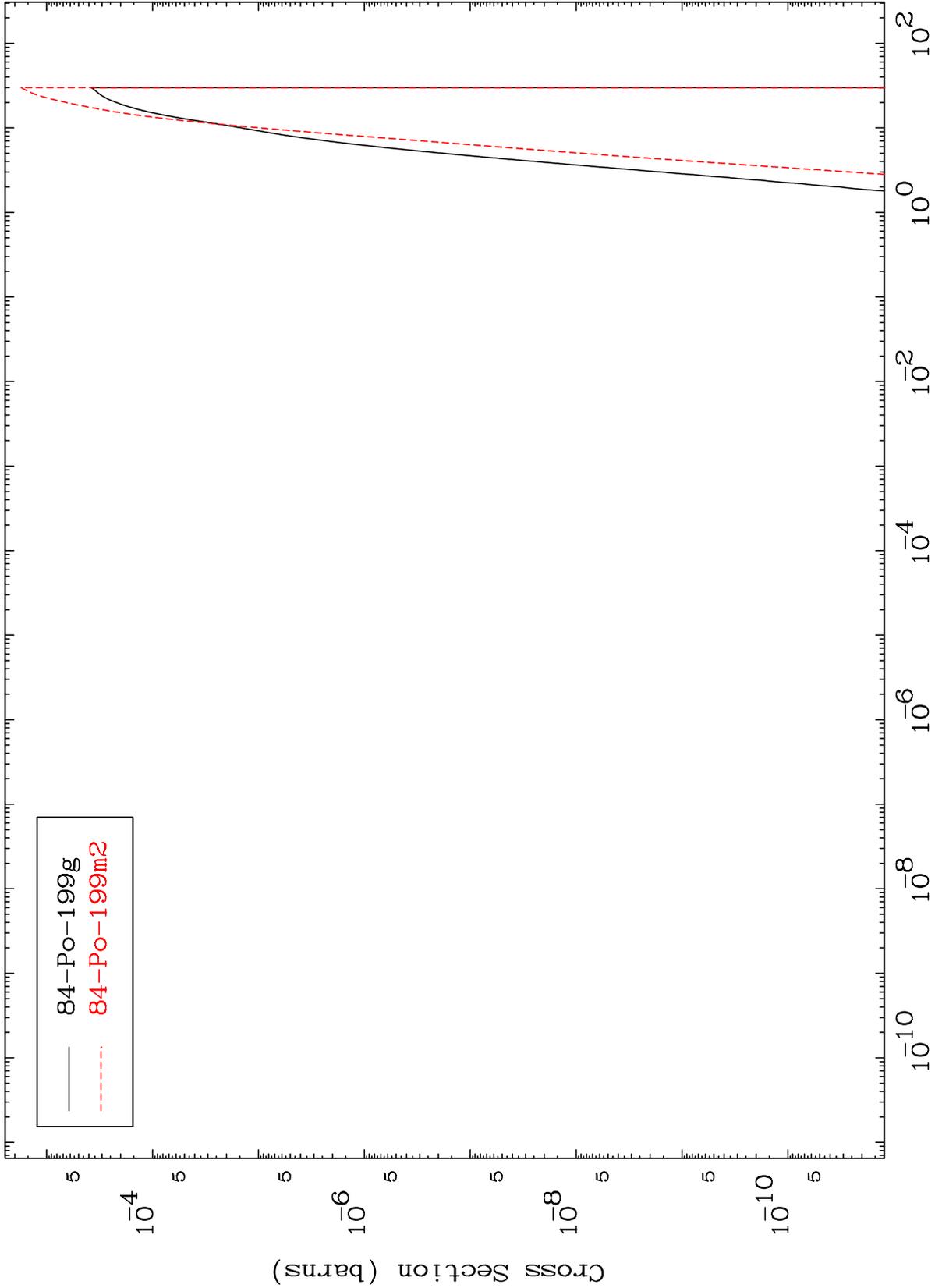


MAT 8592

(n,2p)

86-Rn-200

Radionuclide Production Cross Section



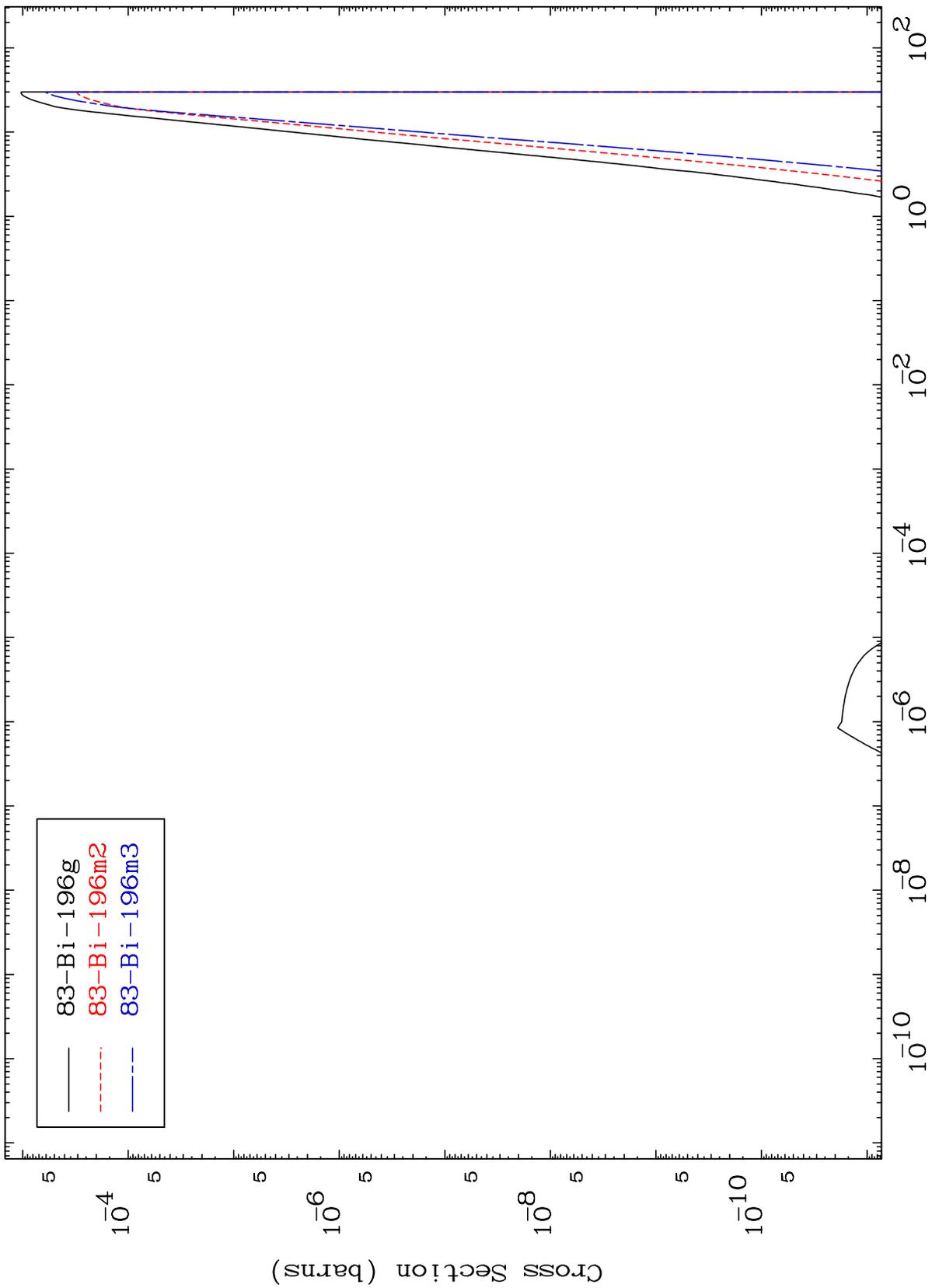
84-Po-199g
84-Po-199m2

MAT 8592

(n,p) α

86-Rn-200

Radionuclide Production Cross Section



46

Incident Energy (MeV)

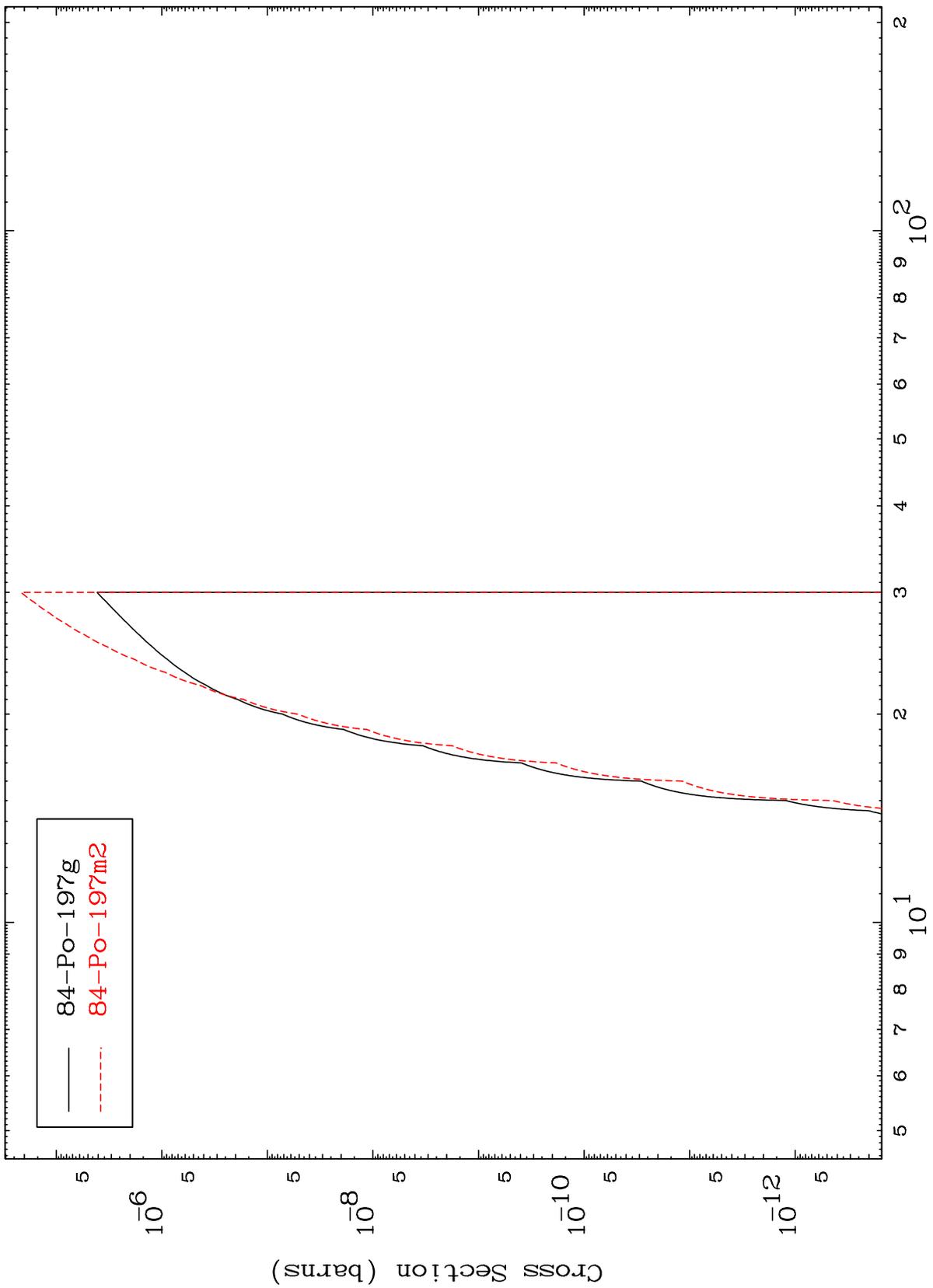
86-Rn-200

MAT 8592

(n,p) t

86-Rn-200

Radionuclide Production Cross Section



47

Incident Energy (MeV)

86-Rn-200

MAT 8592

(n,d) α

86-Rn-200

Radionuclide Production Cross Section

